

Program EVALPLOT  
(Version 2018-1)

by

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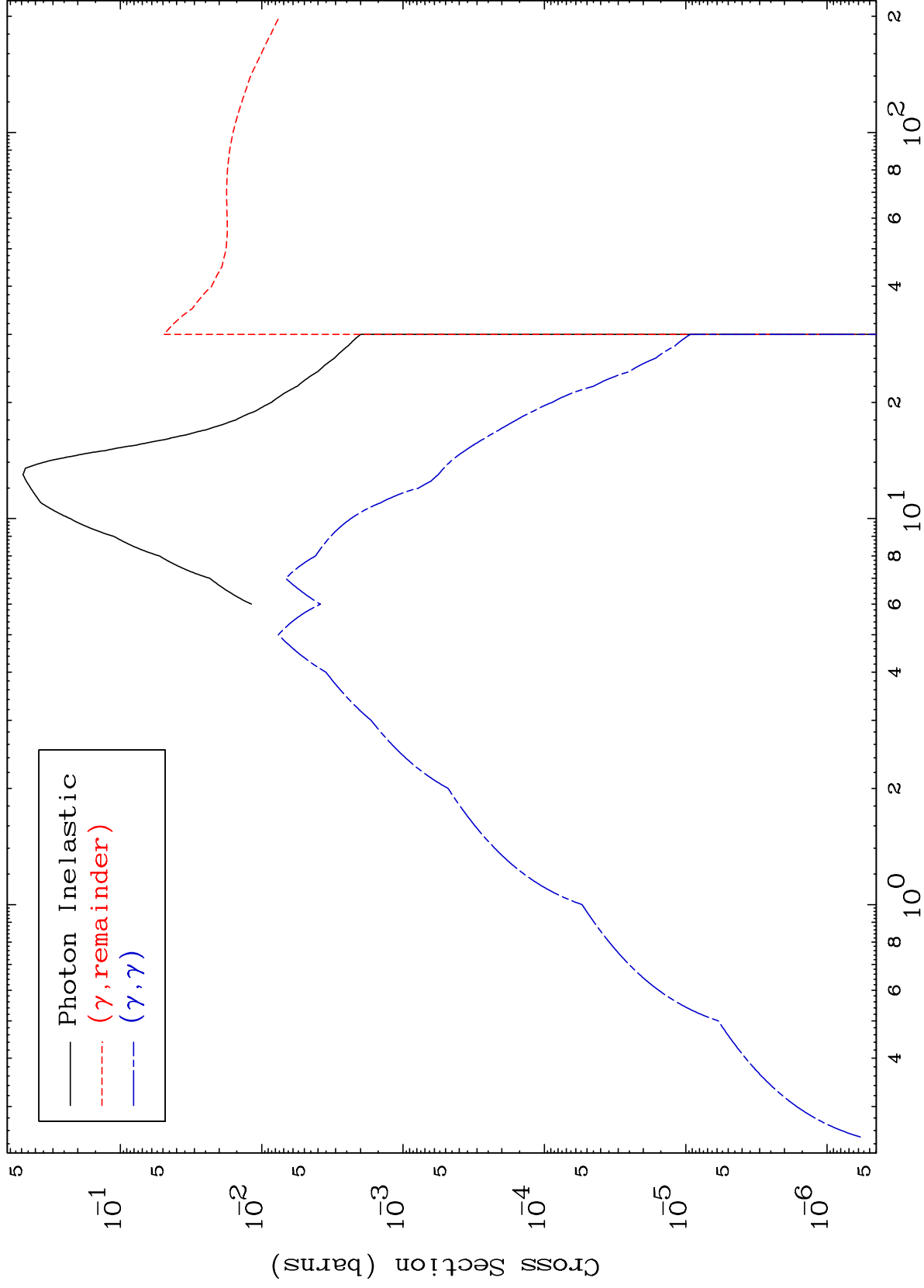
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9955

Photon Major  
0 Kelvin Cross Sections

101-Md-254



Incident Energy (MeV)

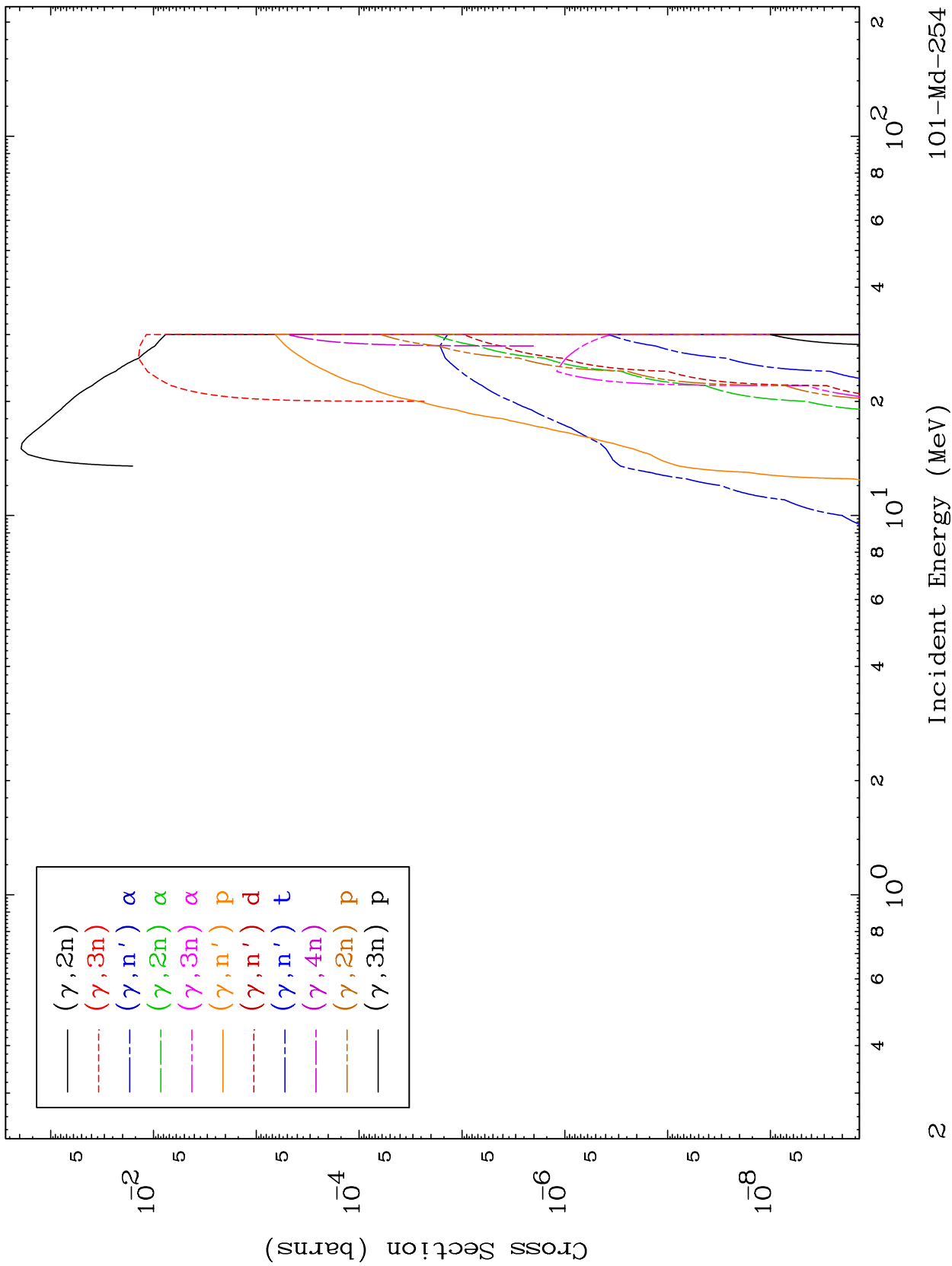
101-Md-254

1

MAT 9955

Photon Neutron Production  
0 Kelvin Cross Sections

101-Md-254



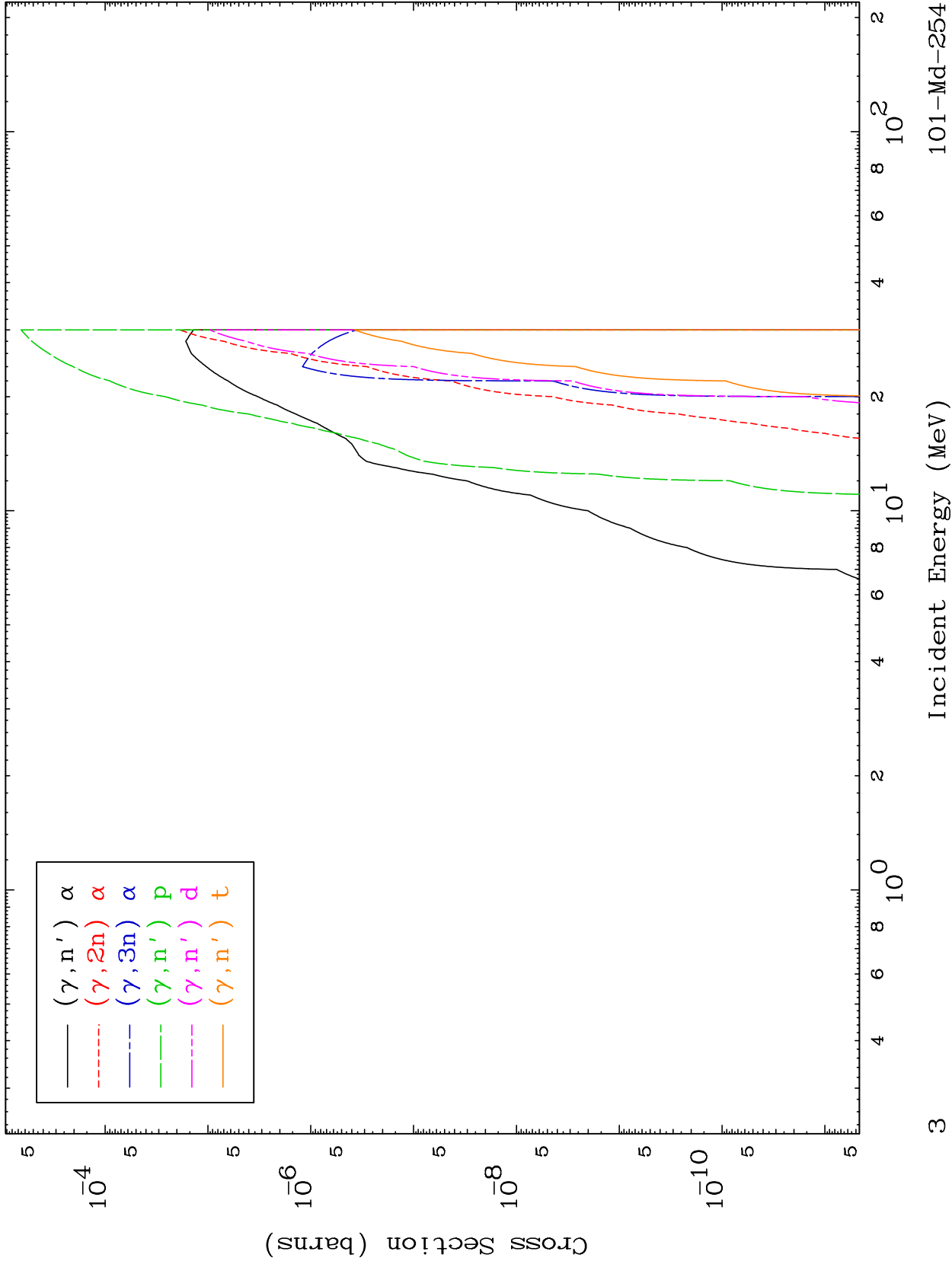
101-Md-254

Incident Energy (MeV)

MAT 9955

Photon Charged Particle  
0 Kelvin Cross Sections

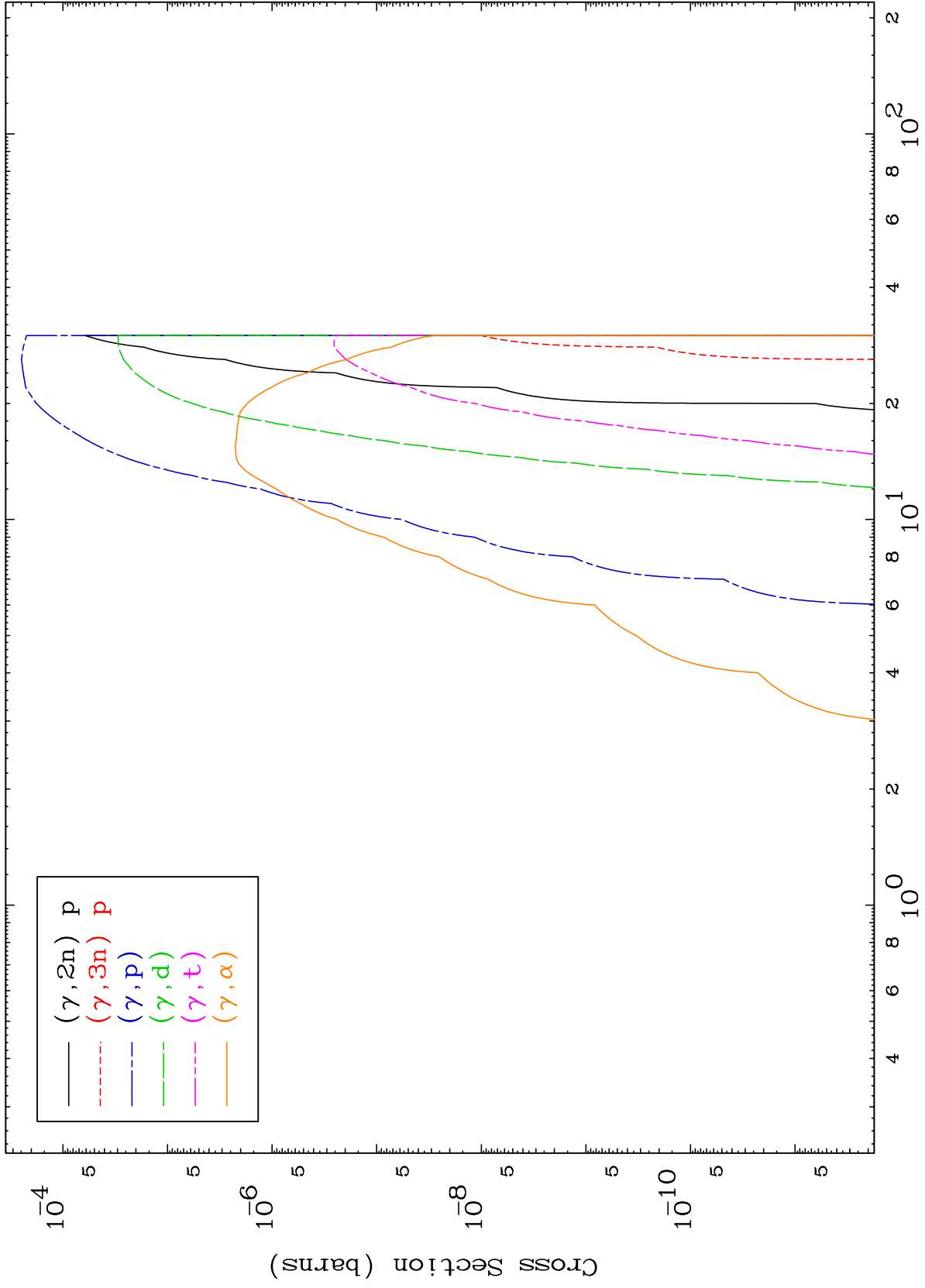
101-Md-254



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Photon Charged Particle  
0 Kelvin Cross Sections

101-Md-254

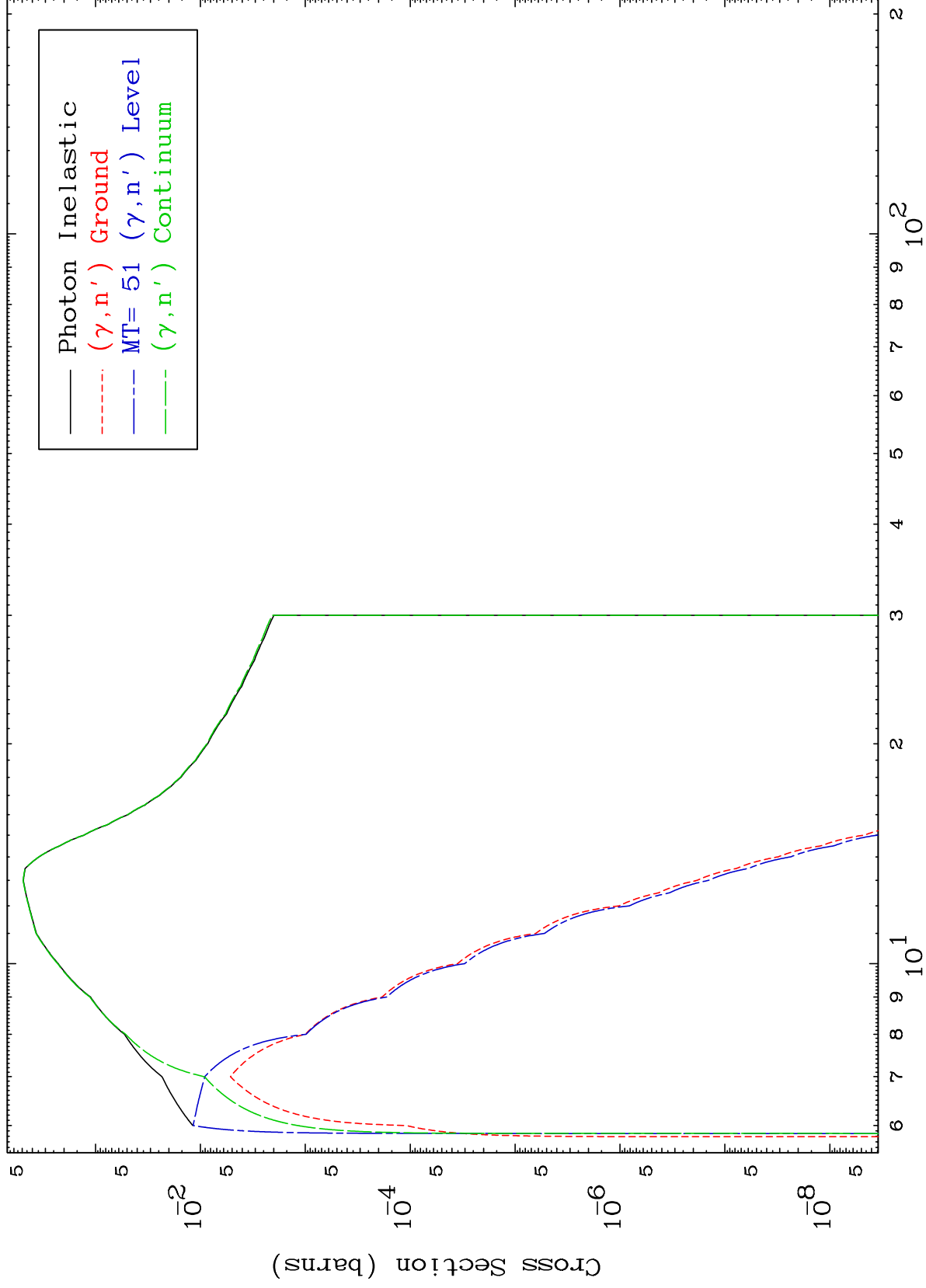


MAT 9955

( $\gamma, n'$ ) Level

101-Md-254

0 Kelvin Cross Sections



5

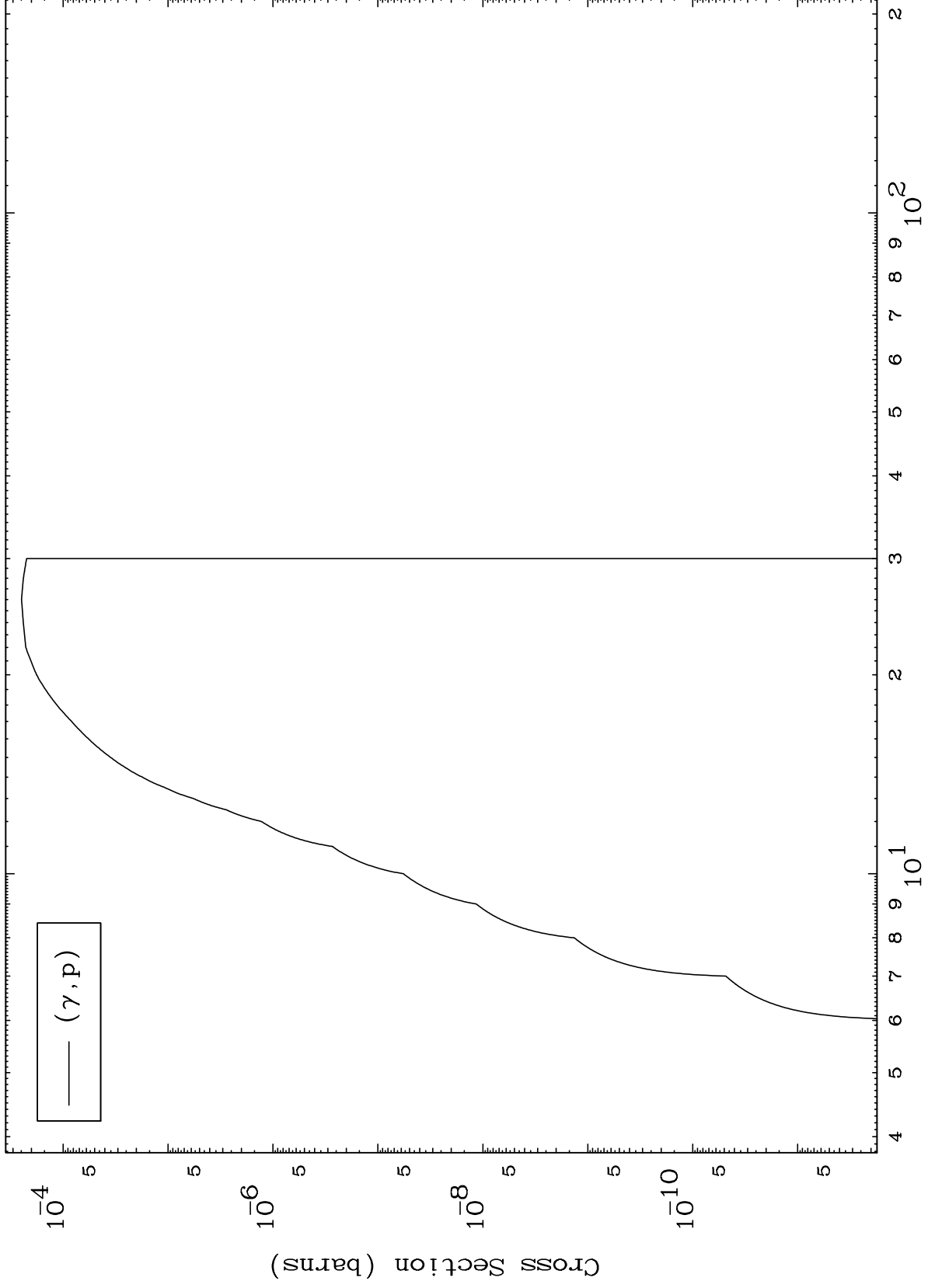
Incident Energy (MeV)

101-Md-254

MAT 9955

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

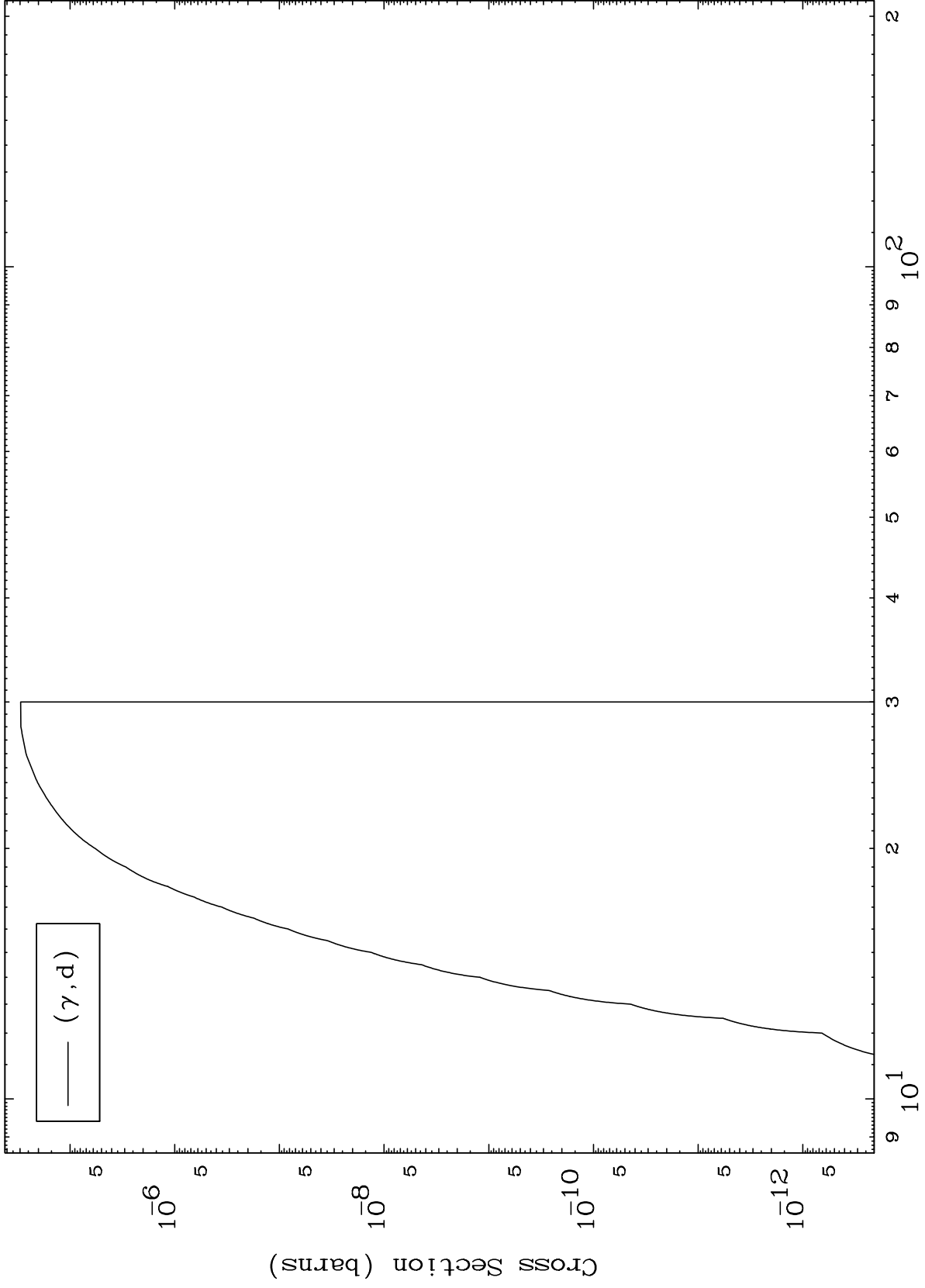
101-Md-254



MAT 9955

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



Incident Energy (MeV)

101-Md-254

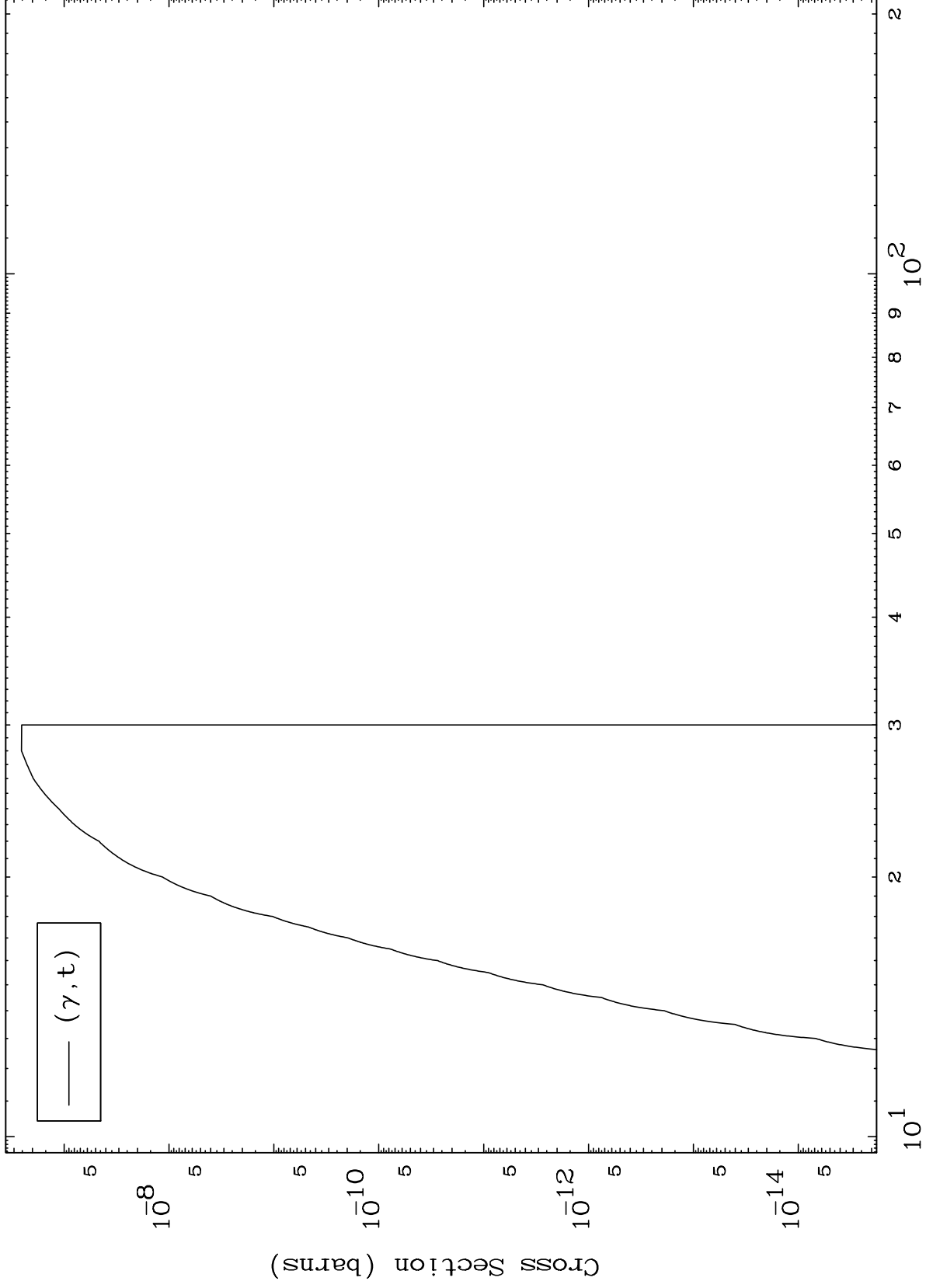
7



MAT 9955

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



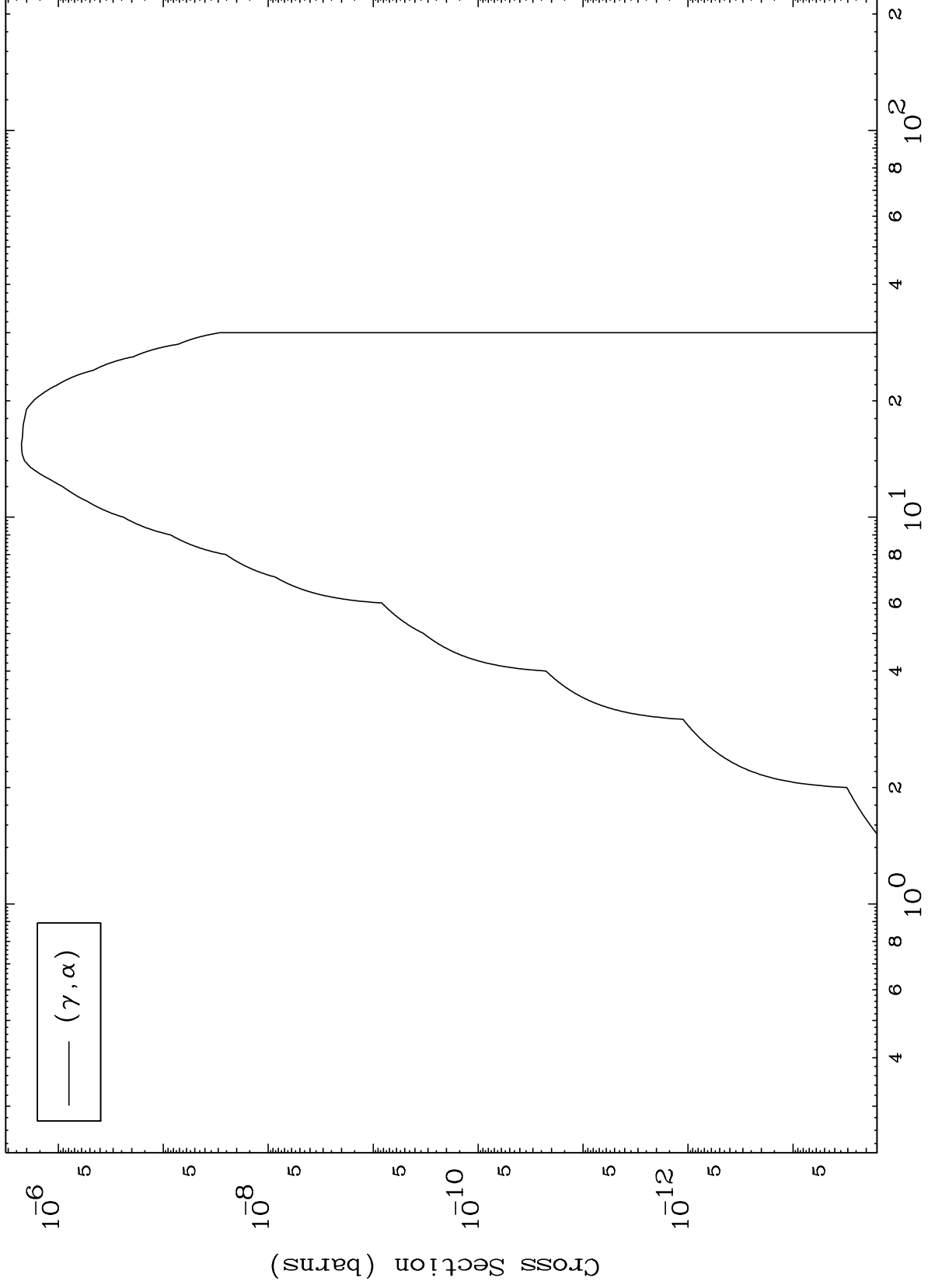
Incident Energy (MeV)

101-Md-254

MAT 9955

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-254

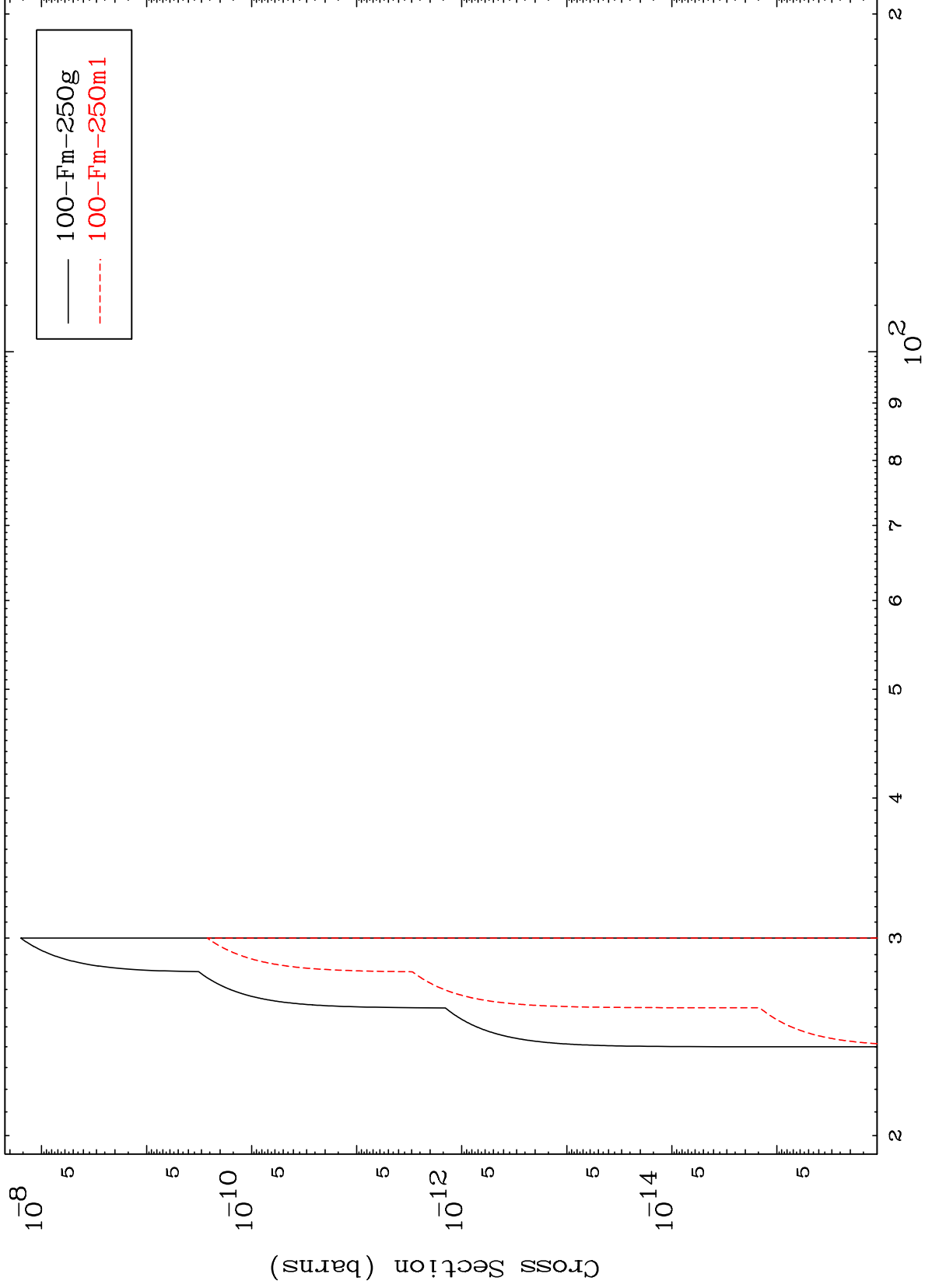


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( $\gamma, 2n$ ) d

101-Md-254

Radionuclide Production Cross Section



10

Incident Energy (MeV)

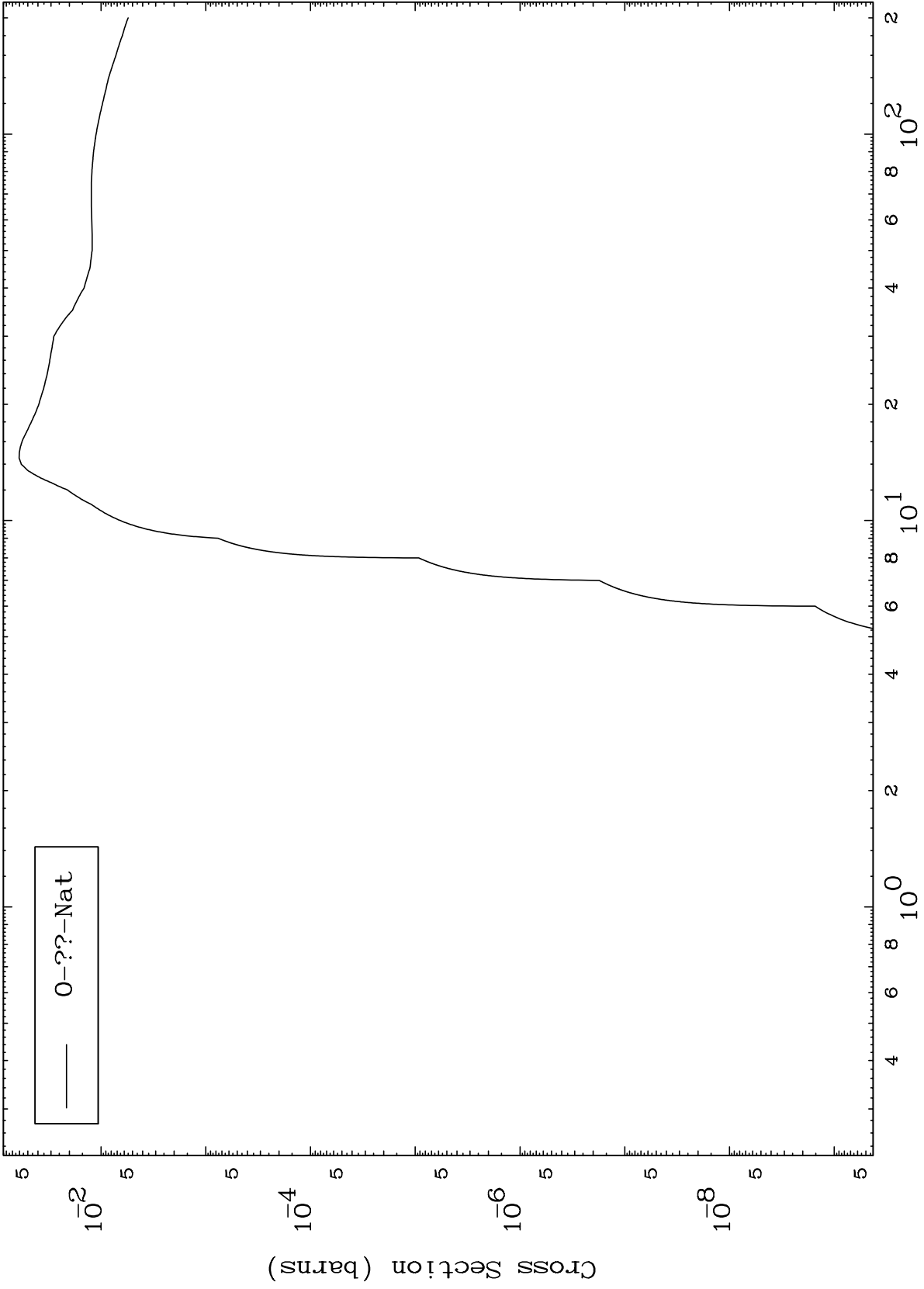
$10^2$

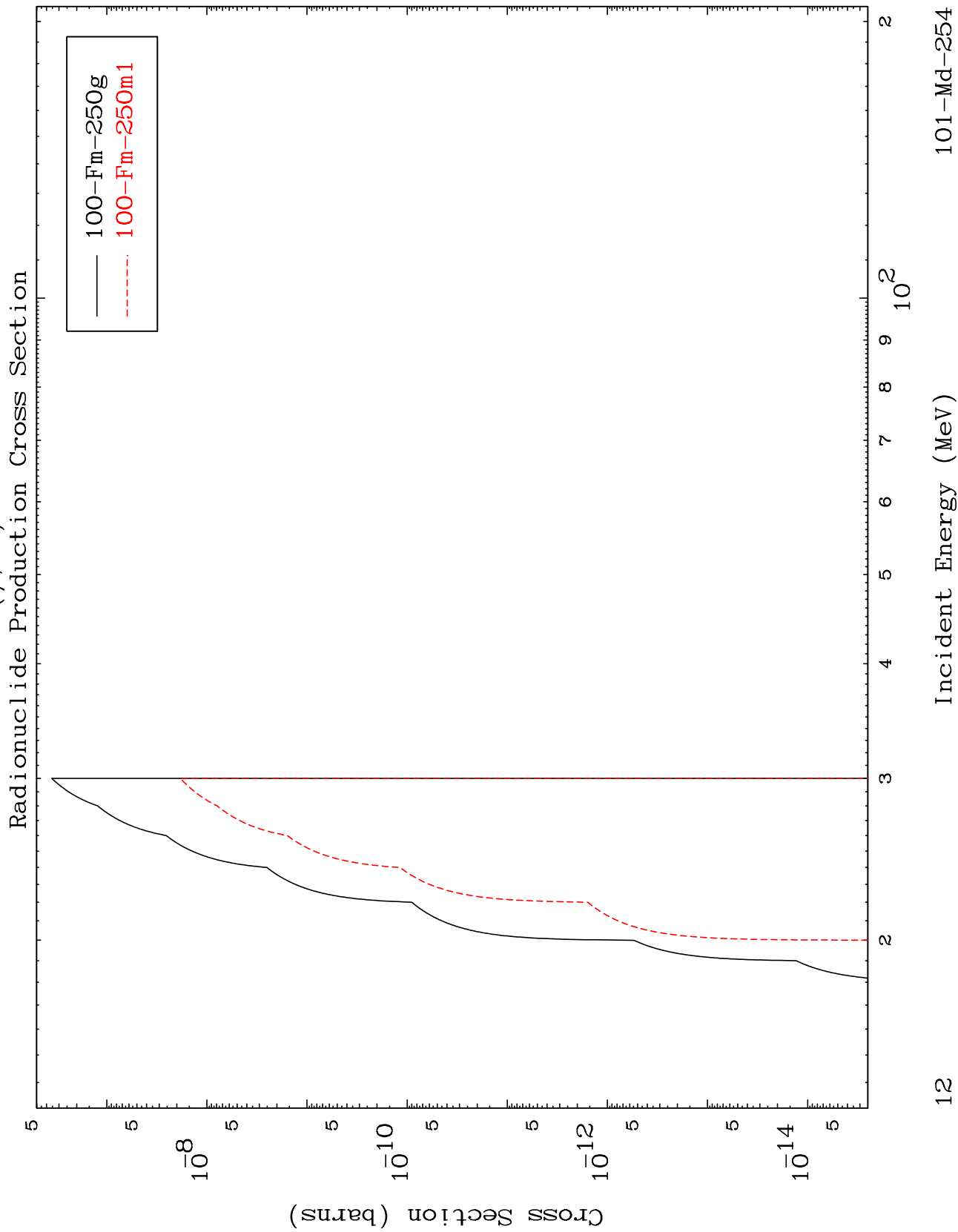
101-Md-254

MAT 9955

101-Md-254

Photon Fission  
Radionuclide Production Cross Section



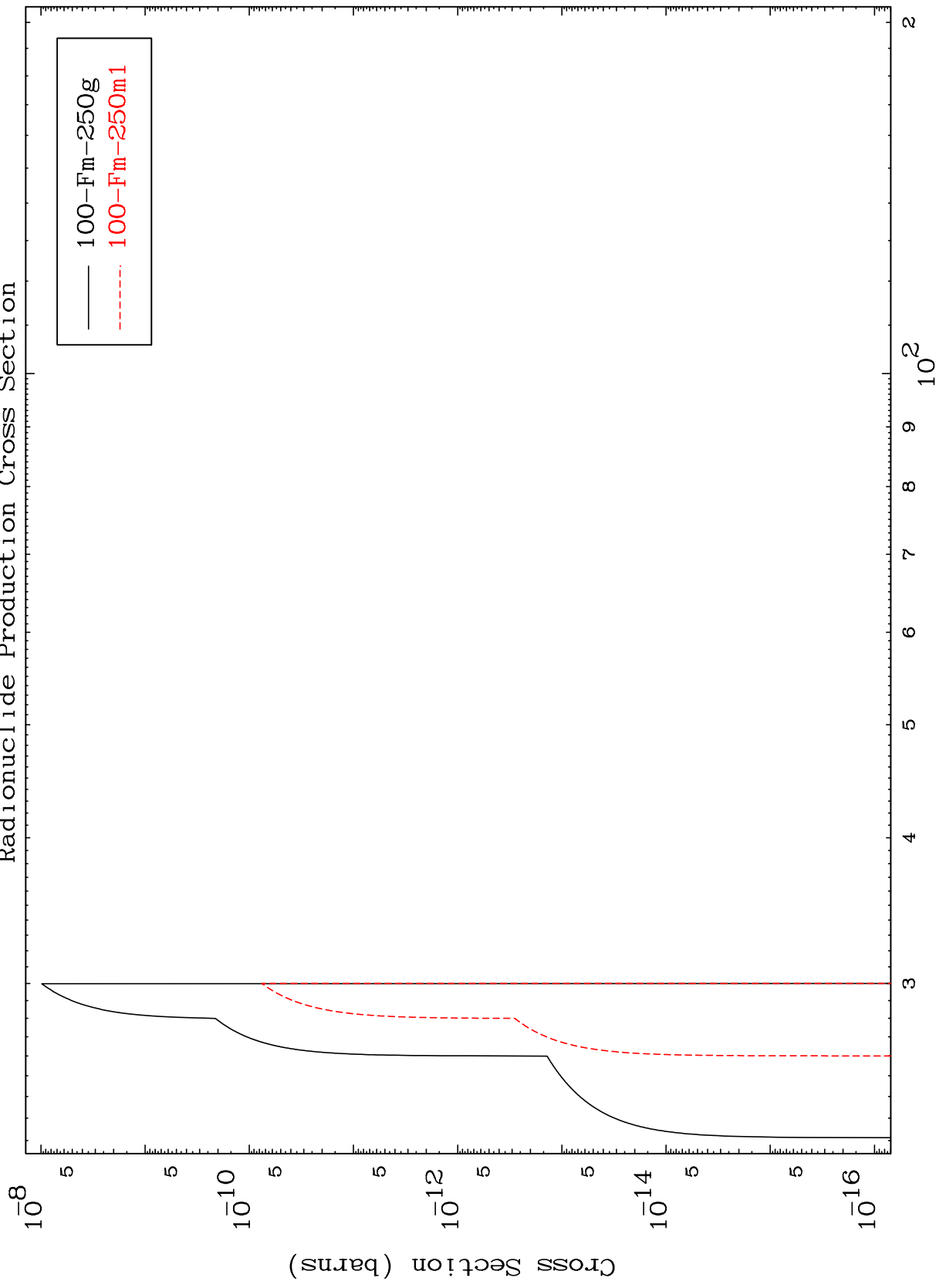


MAT 9955

$(\gamma, 3n)$  p

101-Md-254

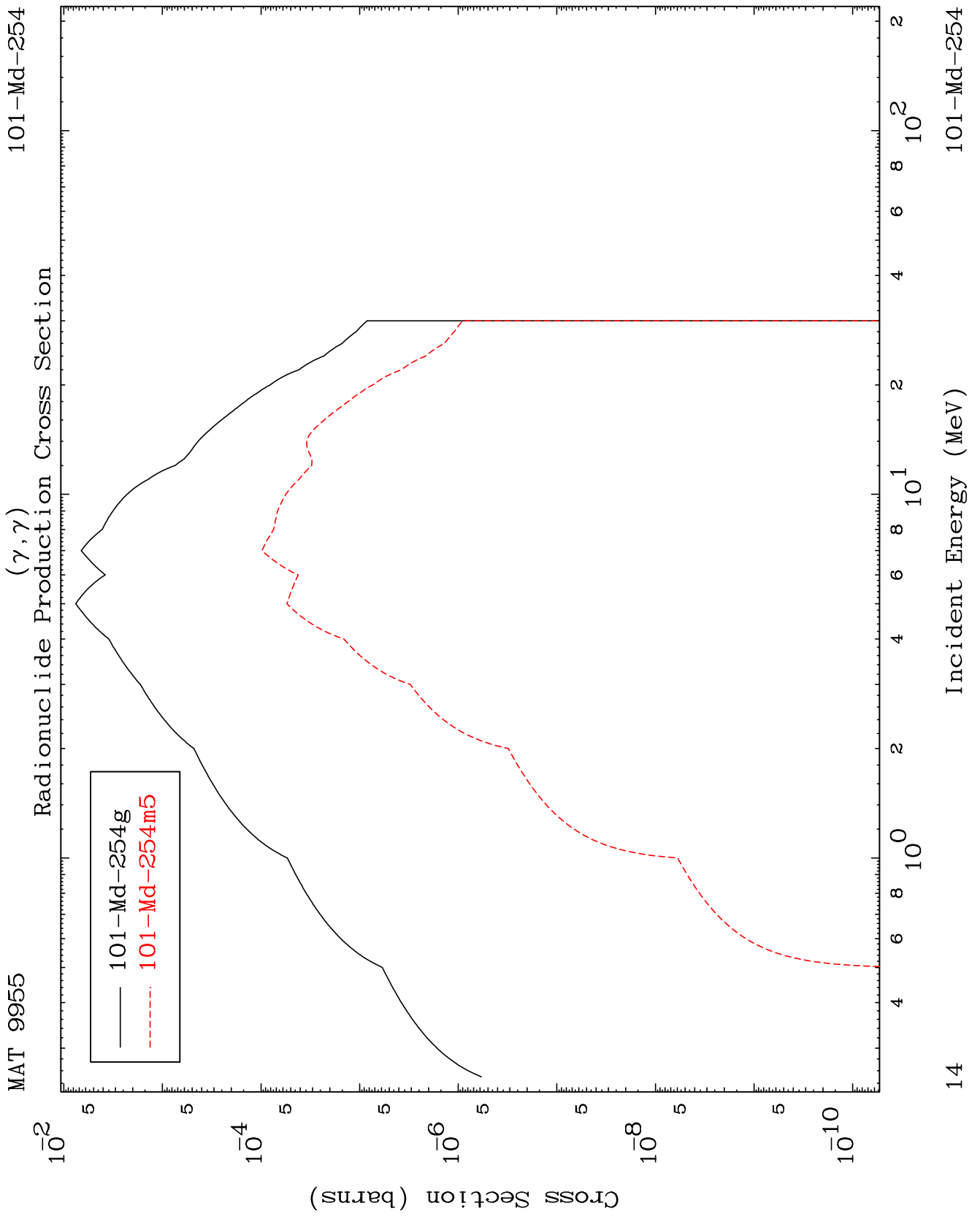
Radionuclide Production Cross Section



13

Incident Energy (MeV)

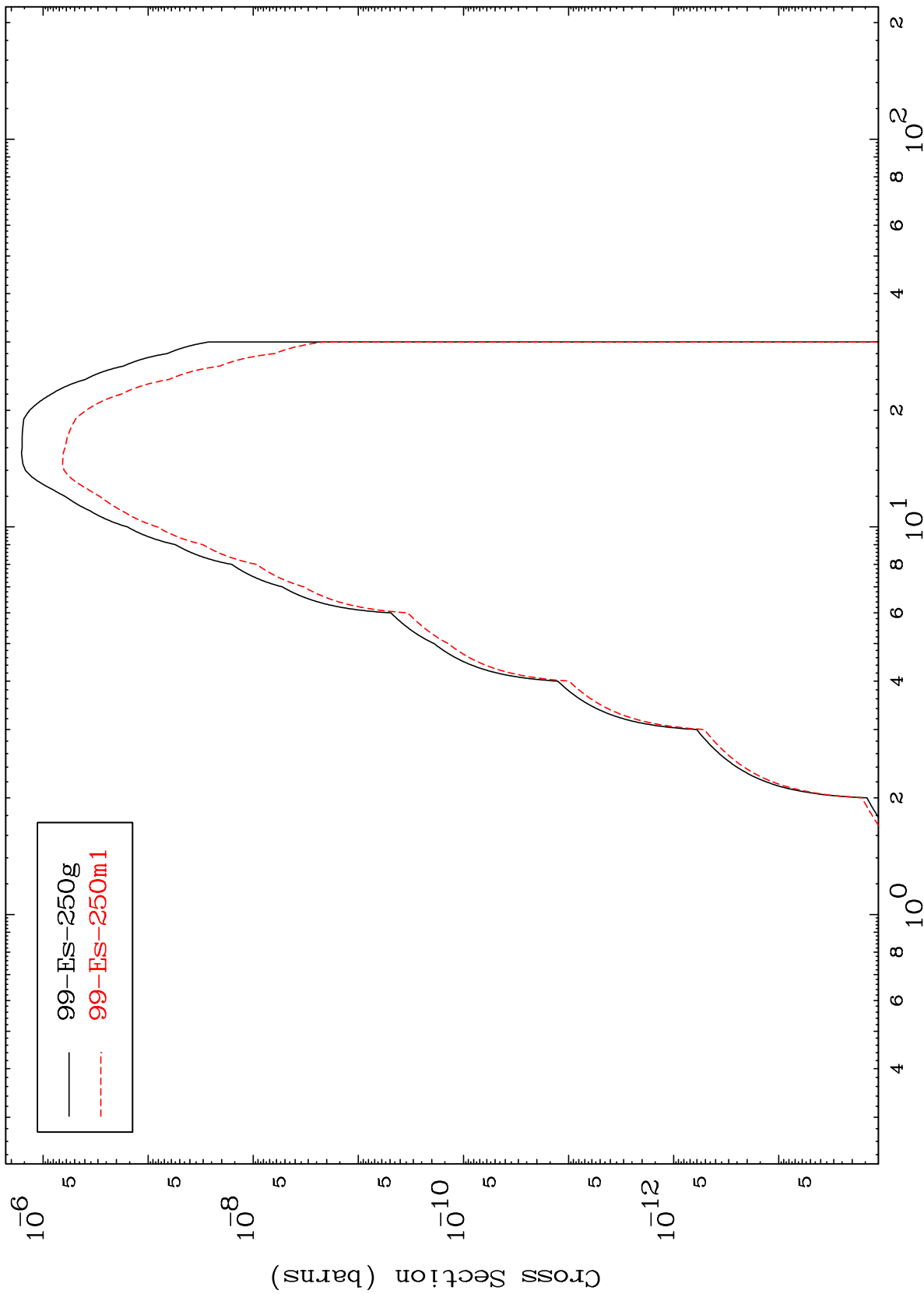
101-Md-254



MAT 9955

101-Md-254

( $\gamma, \alpha$ )  
Radionuclide Production Cross Section



— 99-Es-250g  
- - - 99-Es-250m1

101-Md-254

Incident Energy (MeV)

15