

Program EVALPLOT
(Version 2018-1)

by

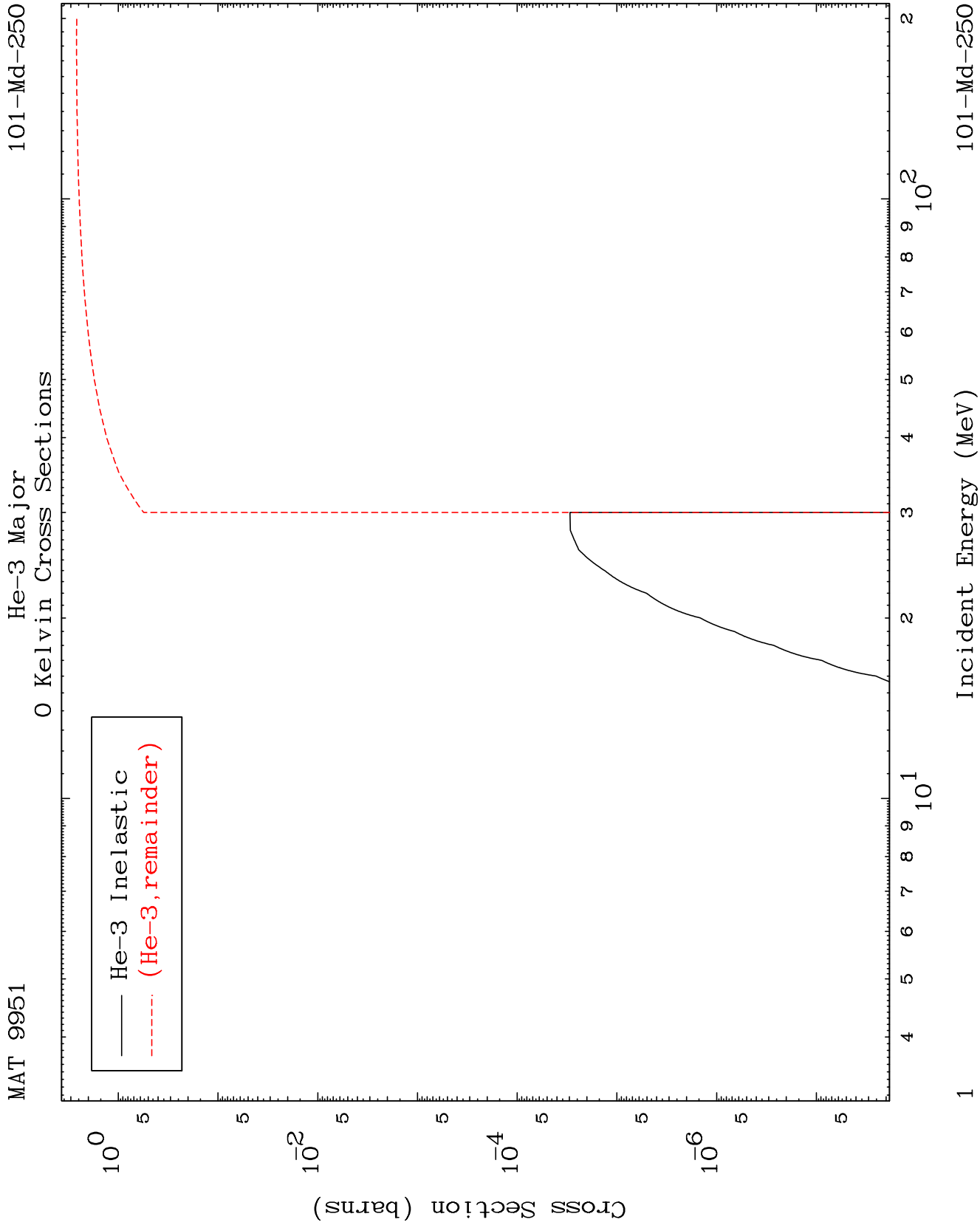
Dermott E. Cullen
(Present Contact Information)

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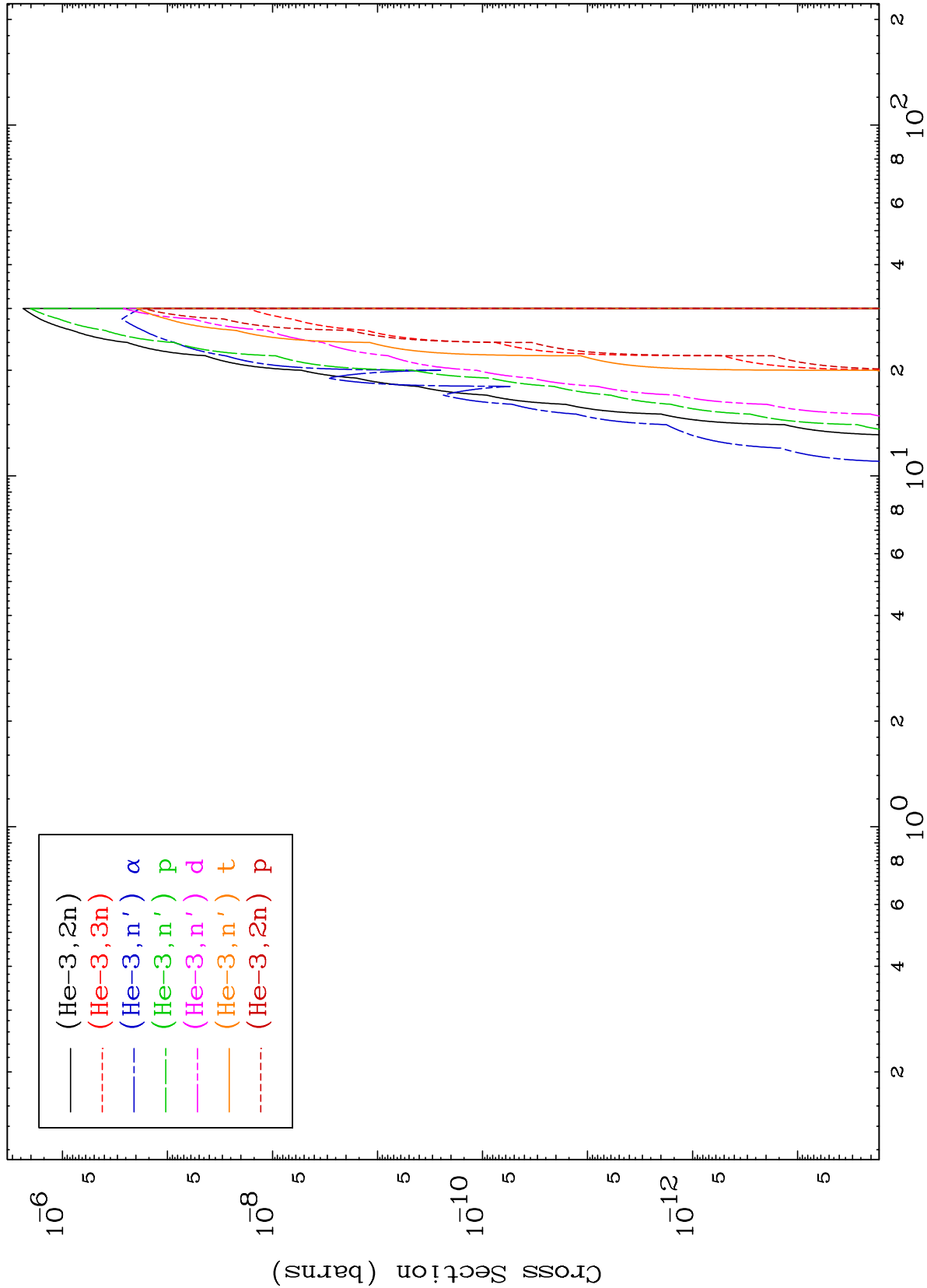
Press Mouse Button to Start

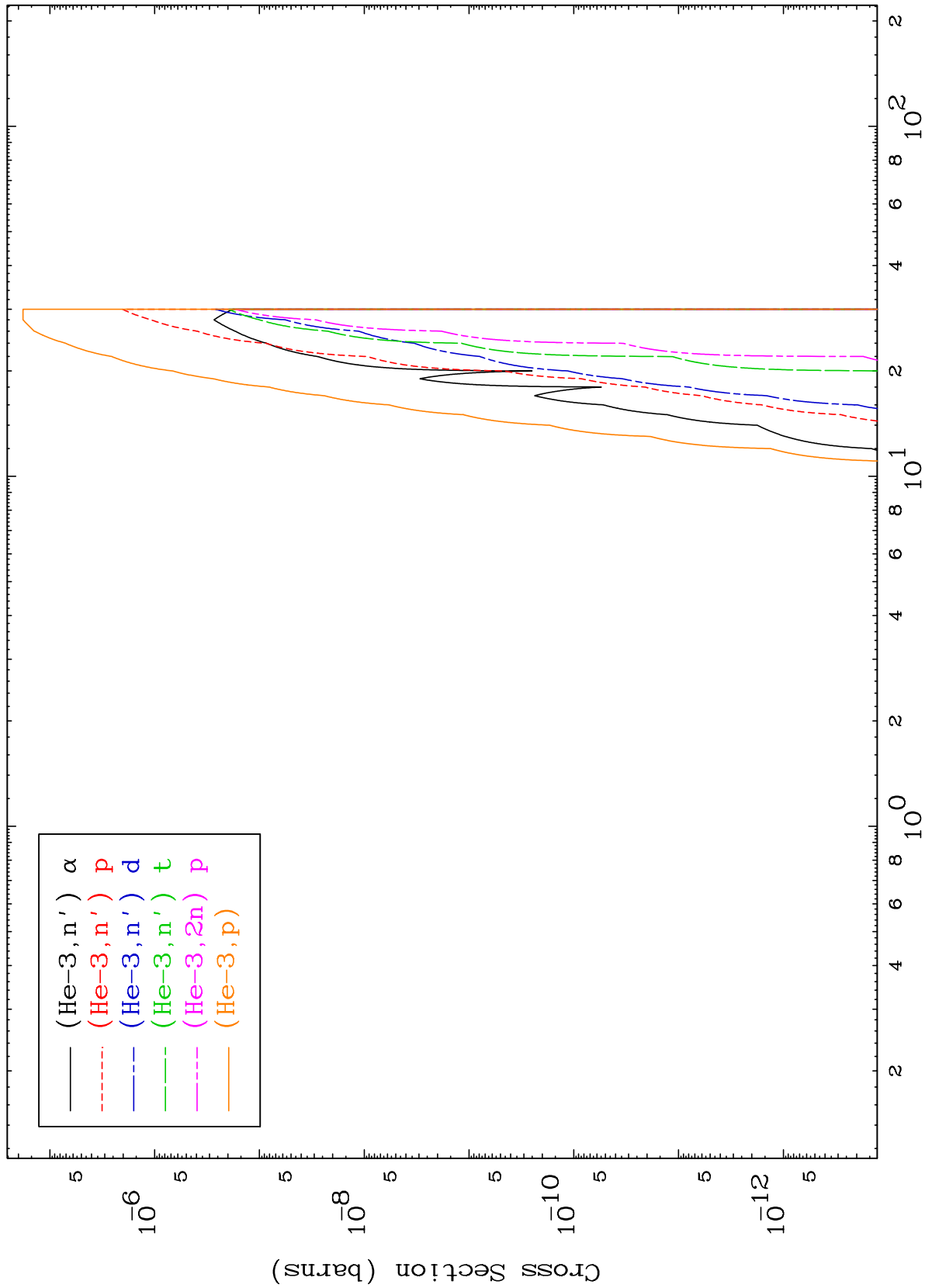


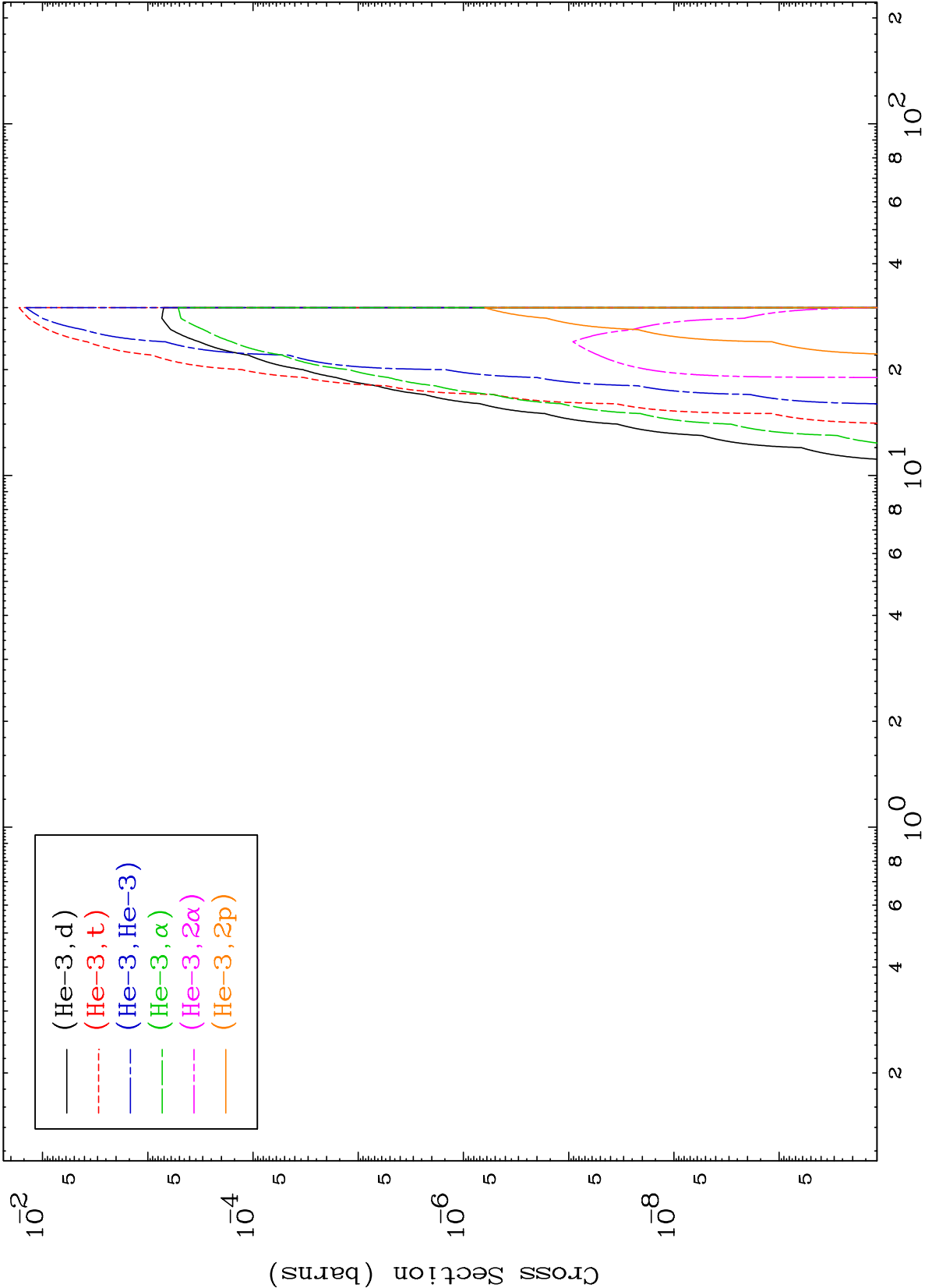
MAT 9951

He-3 Neutron Production
0 Kelvin Cross Sections

101-Md-250



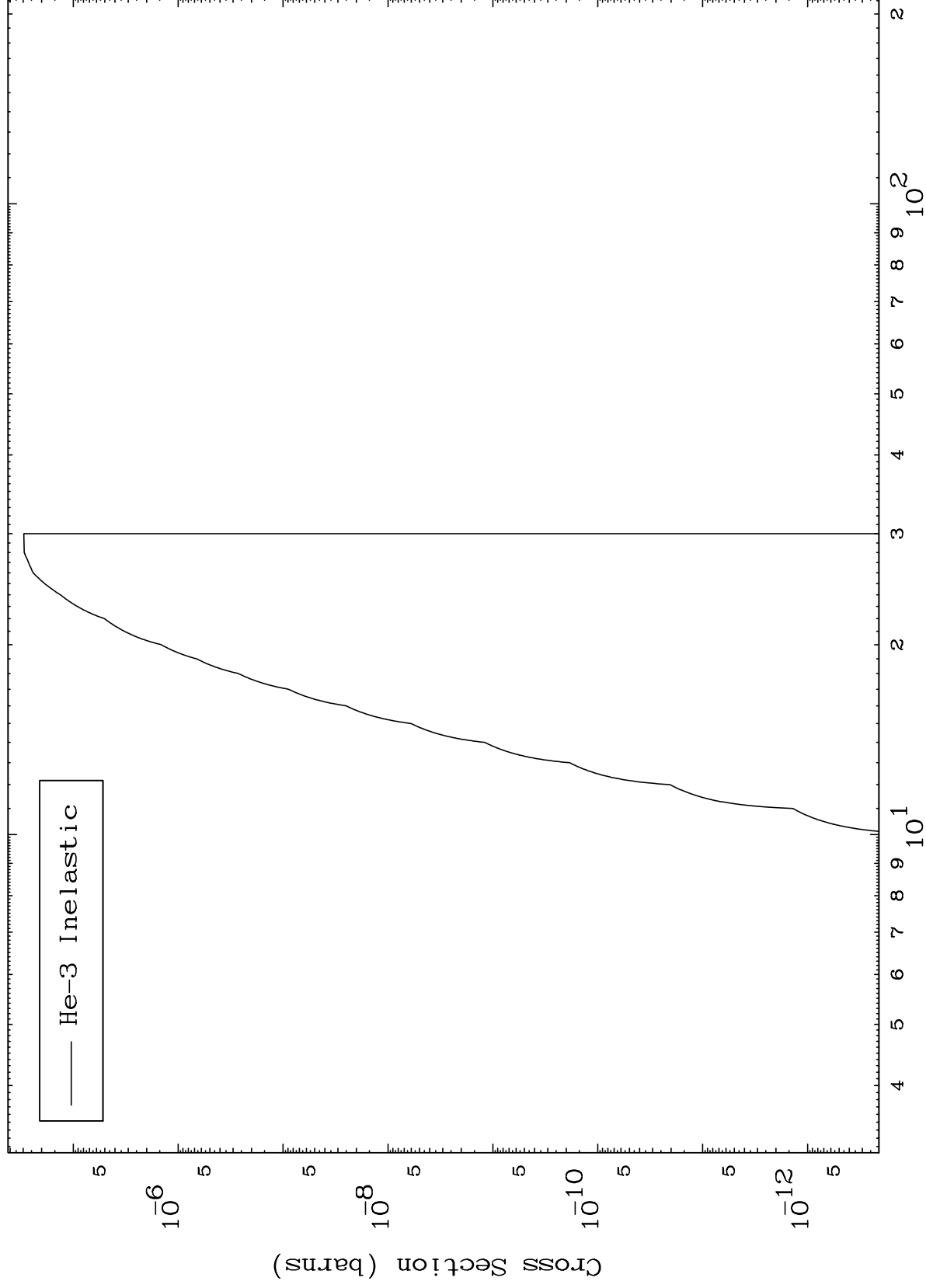




MAT 9951

101-Md-250

(He-3,n') Level
0 Kelvin Cross Sections



101-Md-250

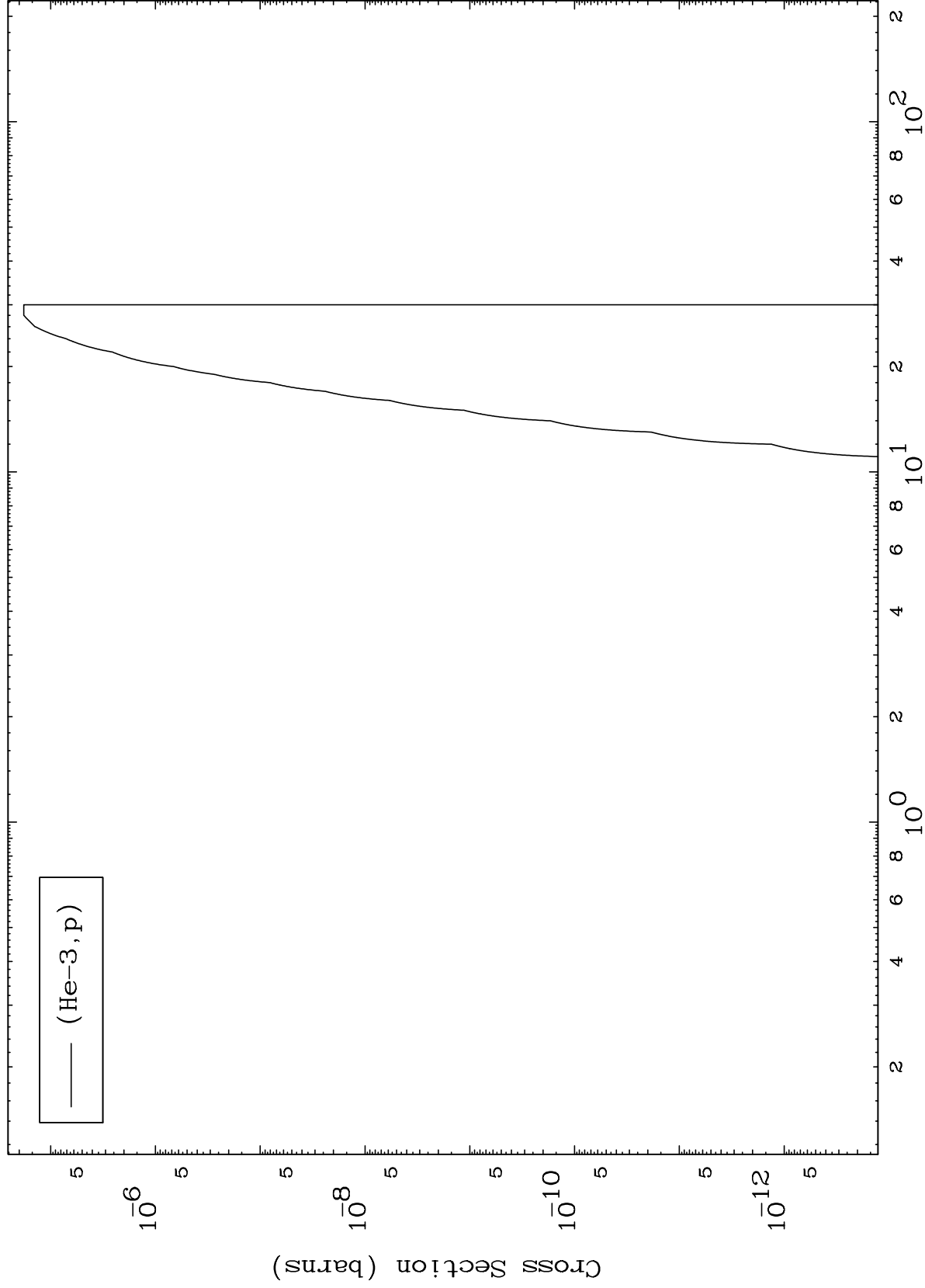
Incident Energy (MeV)

5

MAT 9951

101-Md-250

(He-3,p) Levels
0 Kelvin Cross Sections



101-Md-250

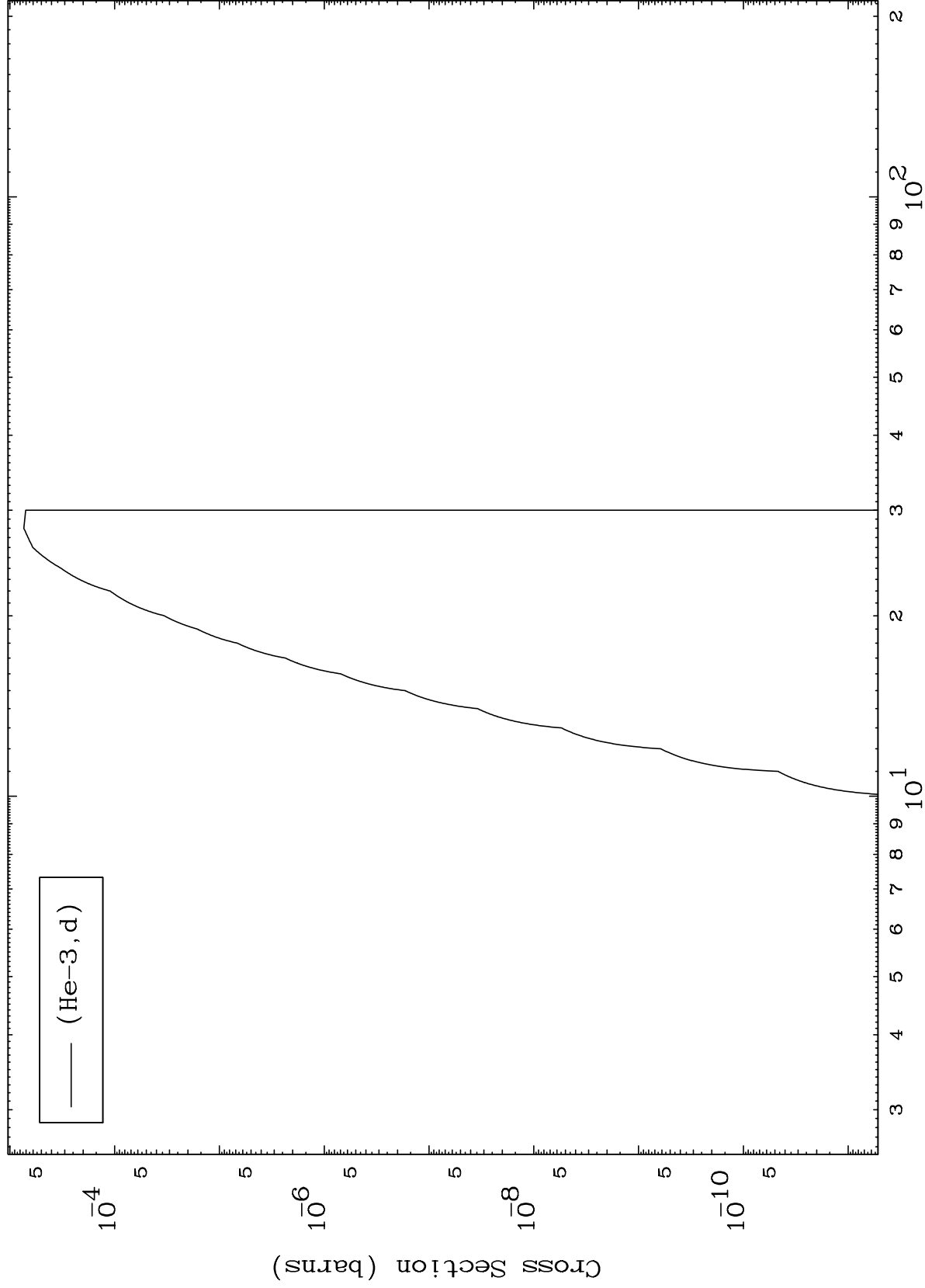
Incident Energy (MeV)

6

MAT 9951

101-Md-250

(He-3,d) Levels
0 Kelvin Cross Sections



101-Md-250

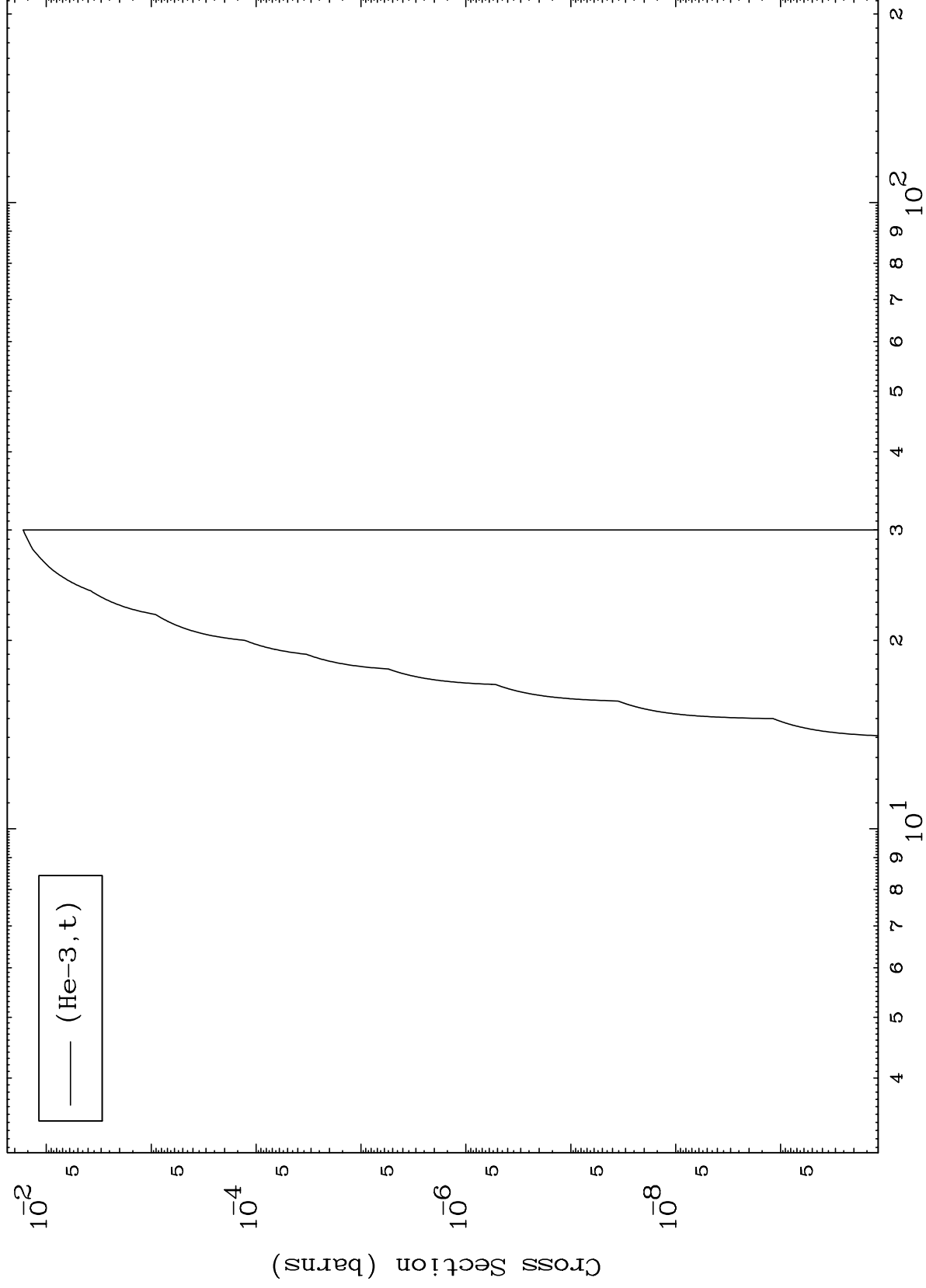
Incident Energy (MeV)

7

MAT 9951

101-Md-250

(He-3,t) Levels
0 Kelvin Cross Sections



8

Incident Energy (MeV)

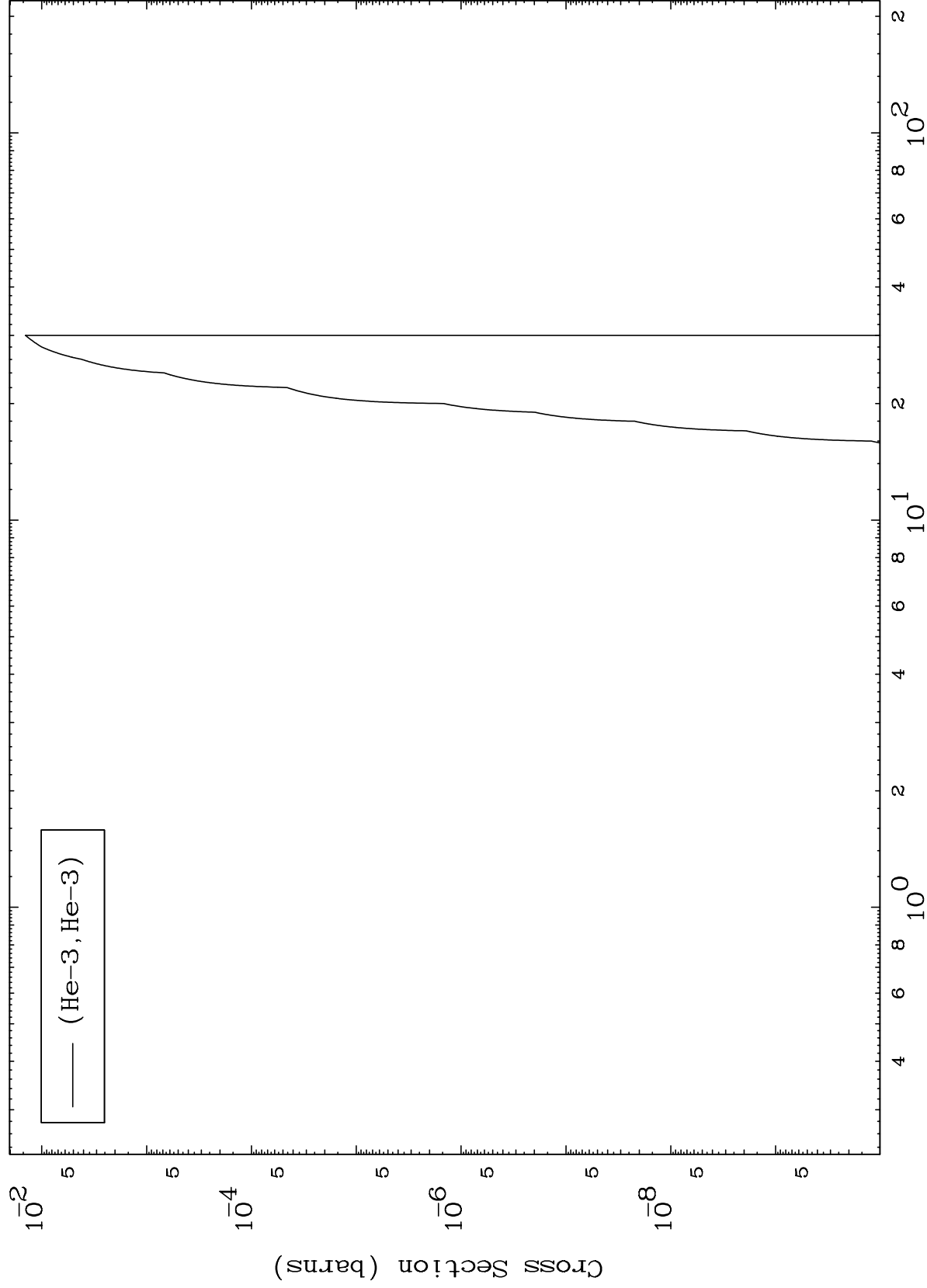
101-Md-250

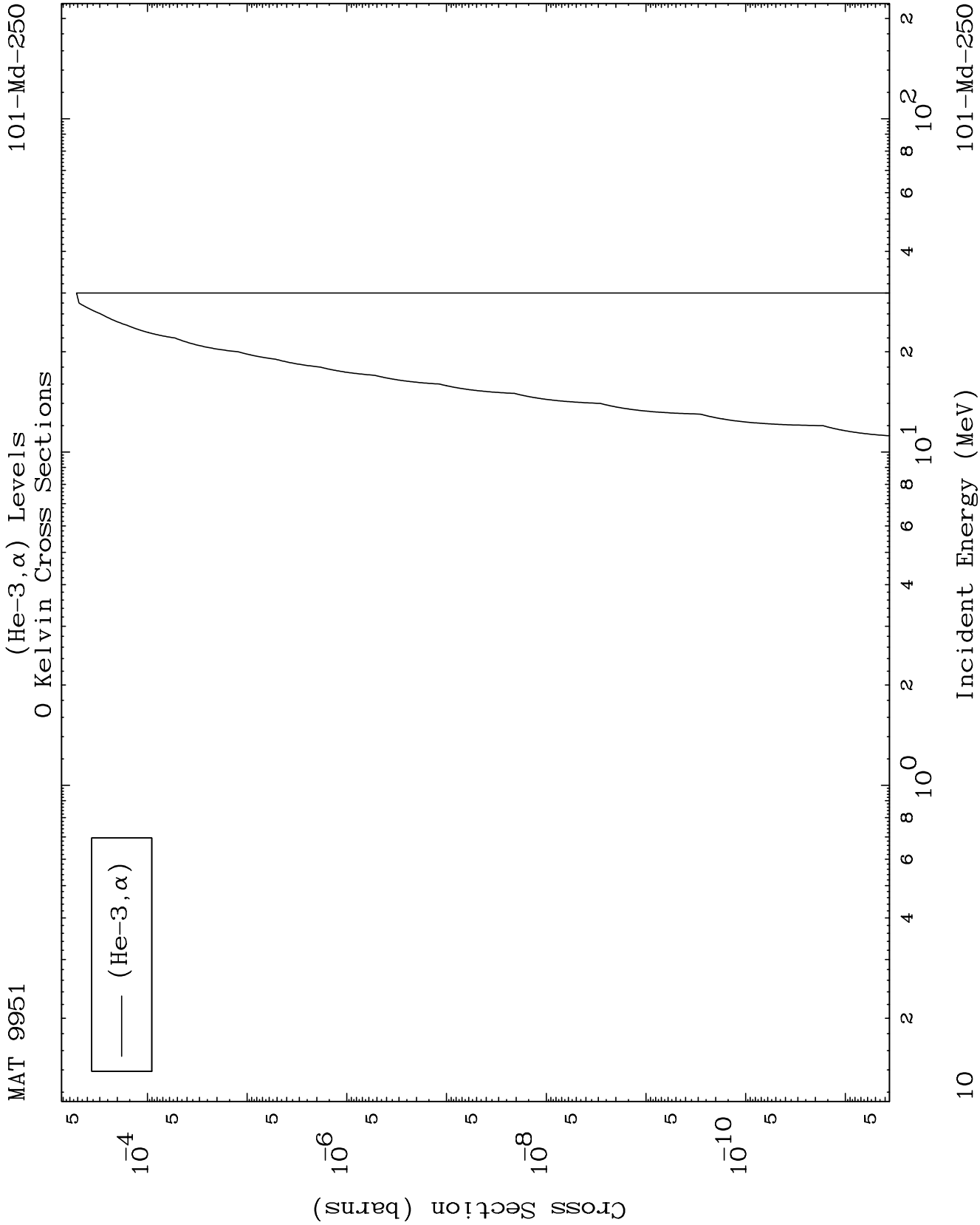
MAT 9951

(He-3, He3) Levels

101-Md-250

0 Kelvin Cross Sections



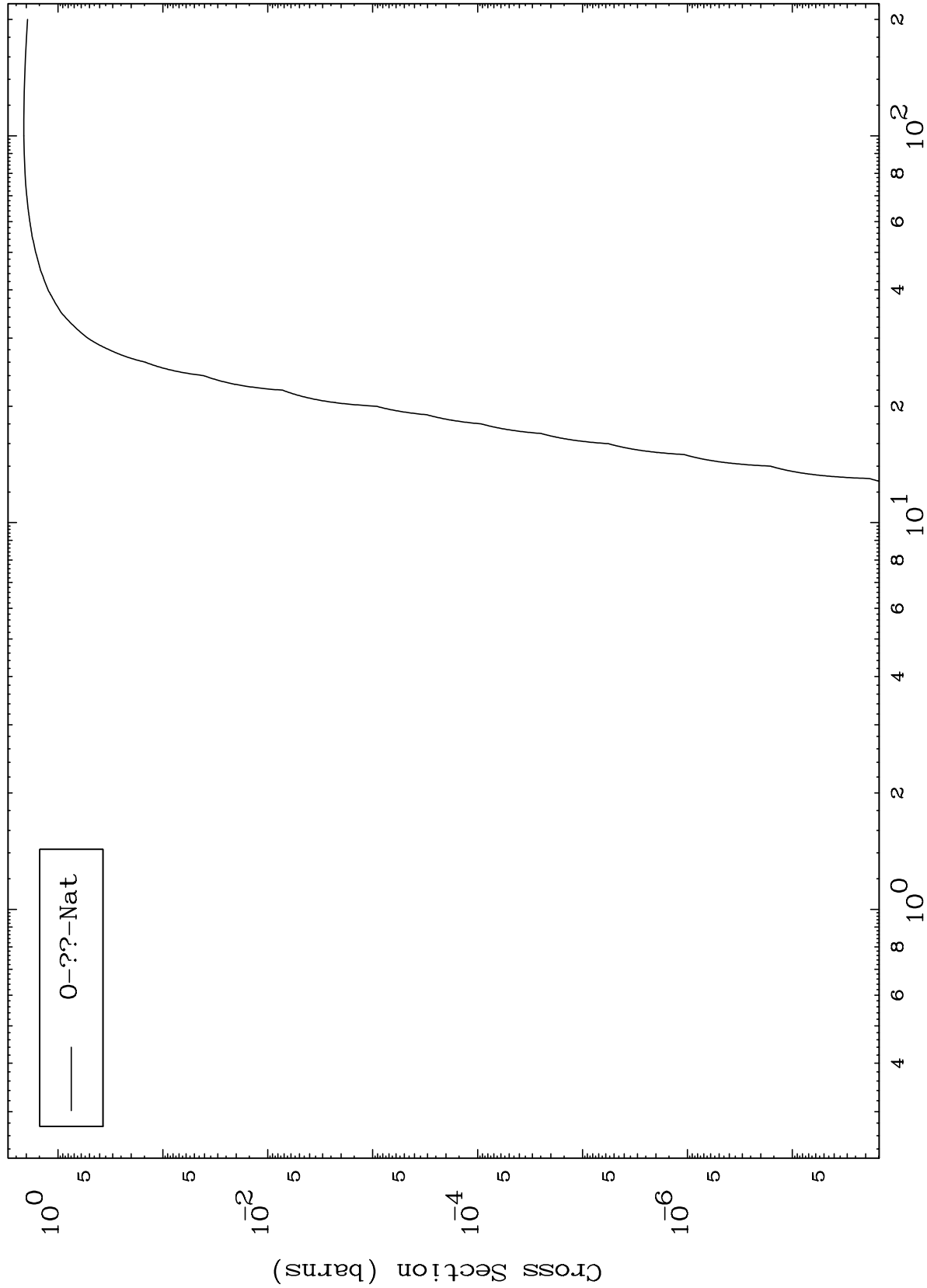


MAT 9951

He-3 Fission

101-Md-250

Radionuclide Production Cross Section

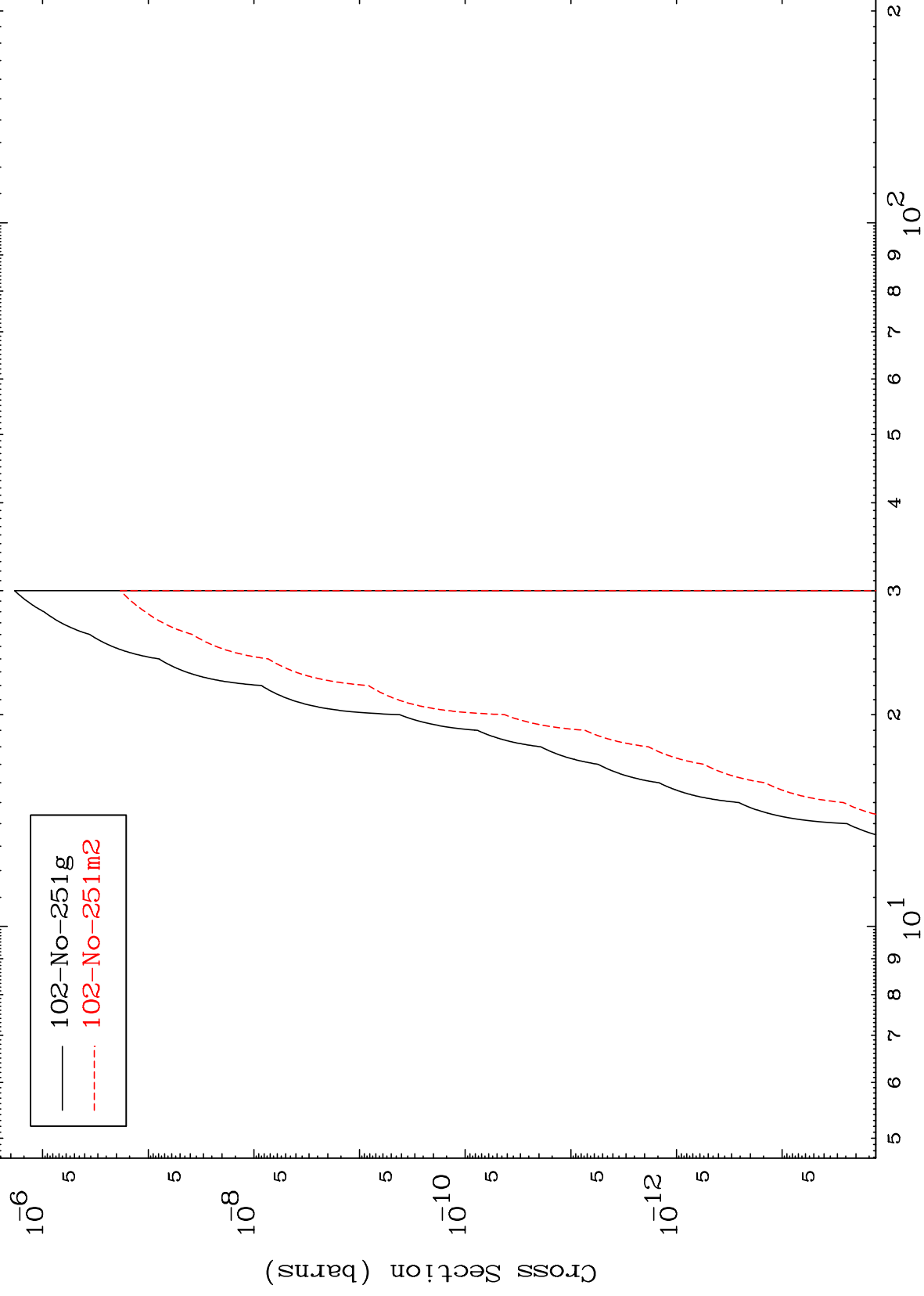


MAT 9951

(He-3,n') p

101-Md-250

Radionuclide Production Cross Section

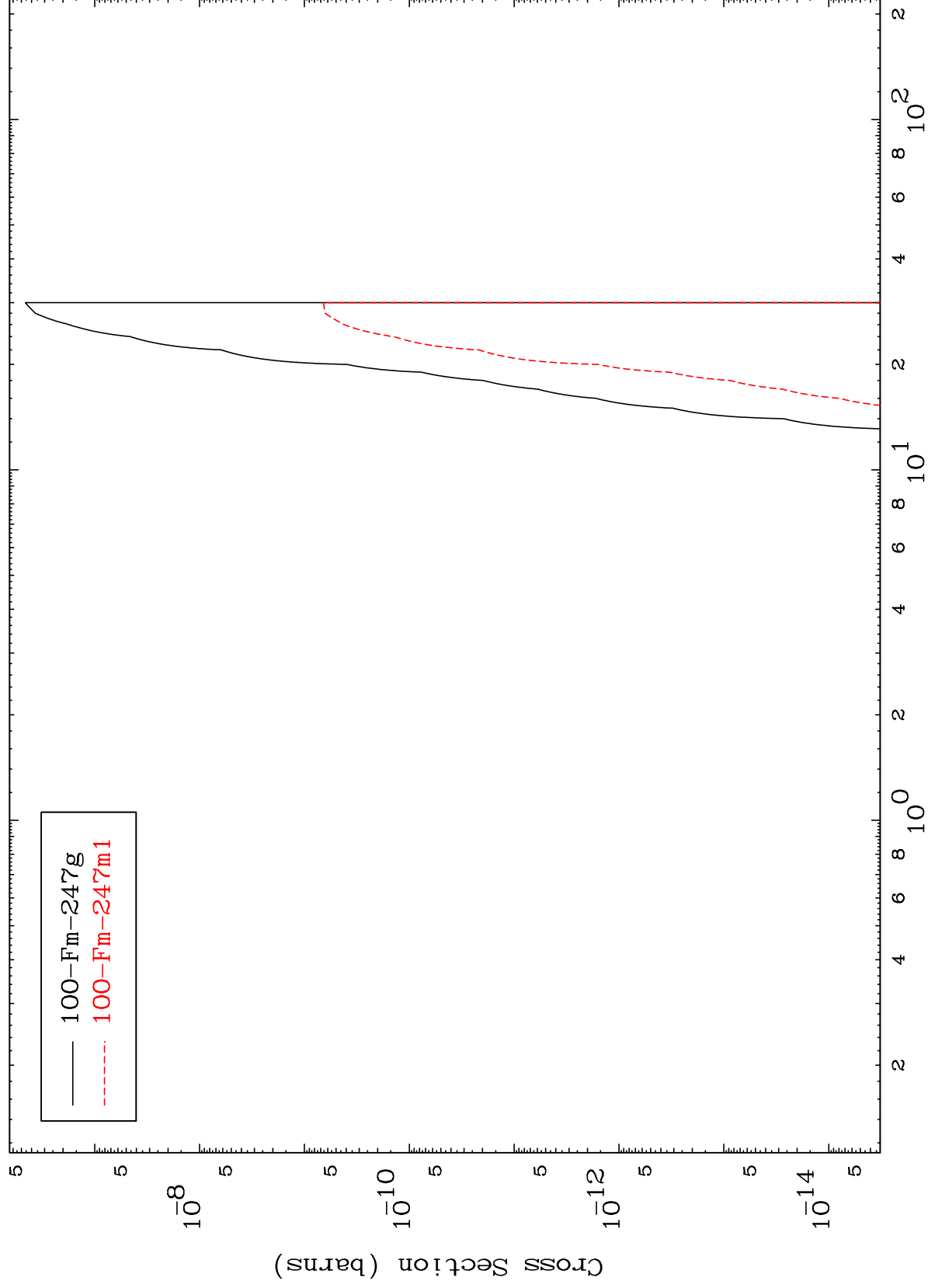


MAT 9951

(He-3,n') p α

101-Md-250

Radionuclide Production Cross Section



13

Incident Energy (MeV)

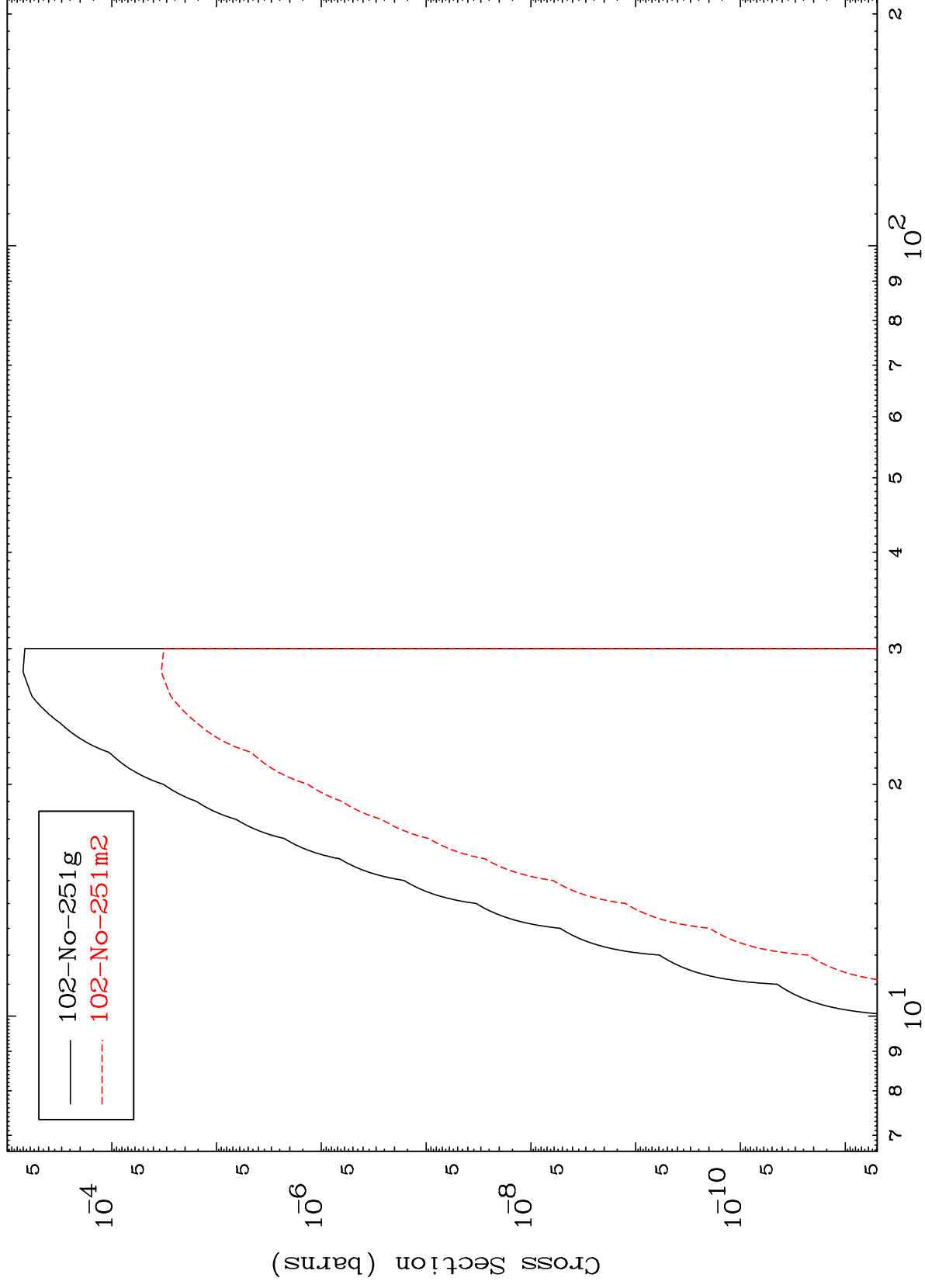
101-Md-250

MAT 9951

(He-3,d)

101-Md-250

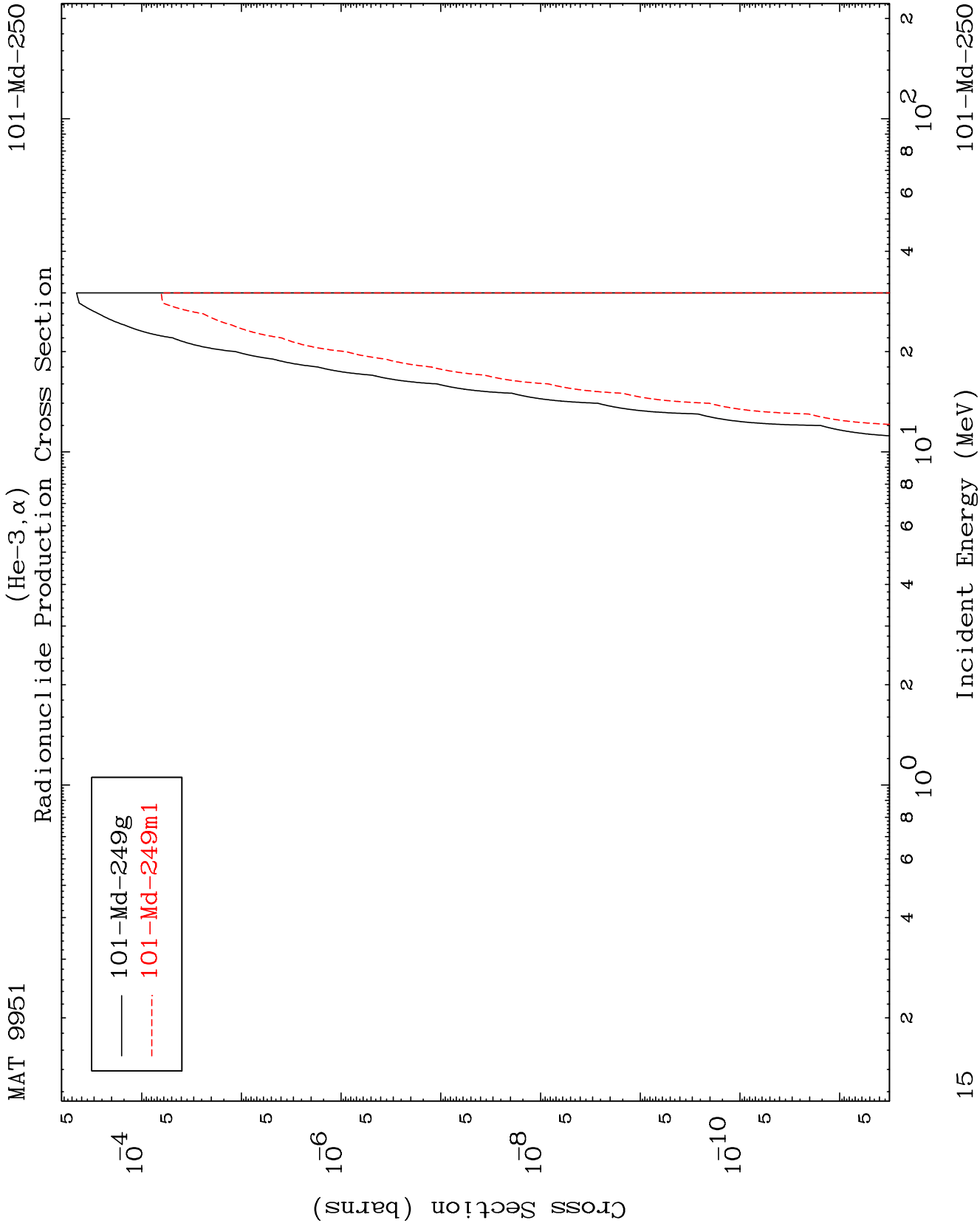
Radionuclide Production Cross Section



14

Incident Energy (MeV)

101-Md-250

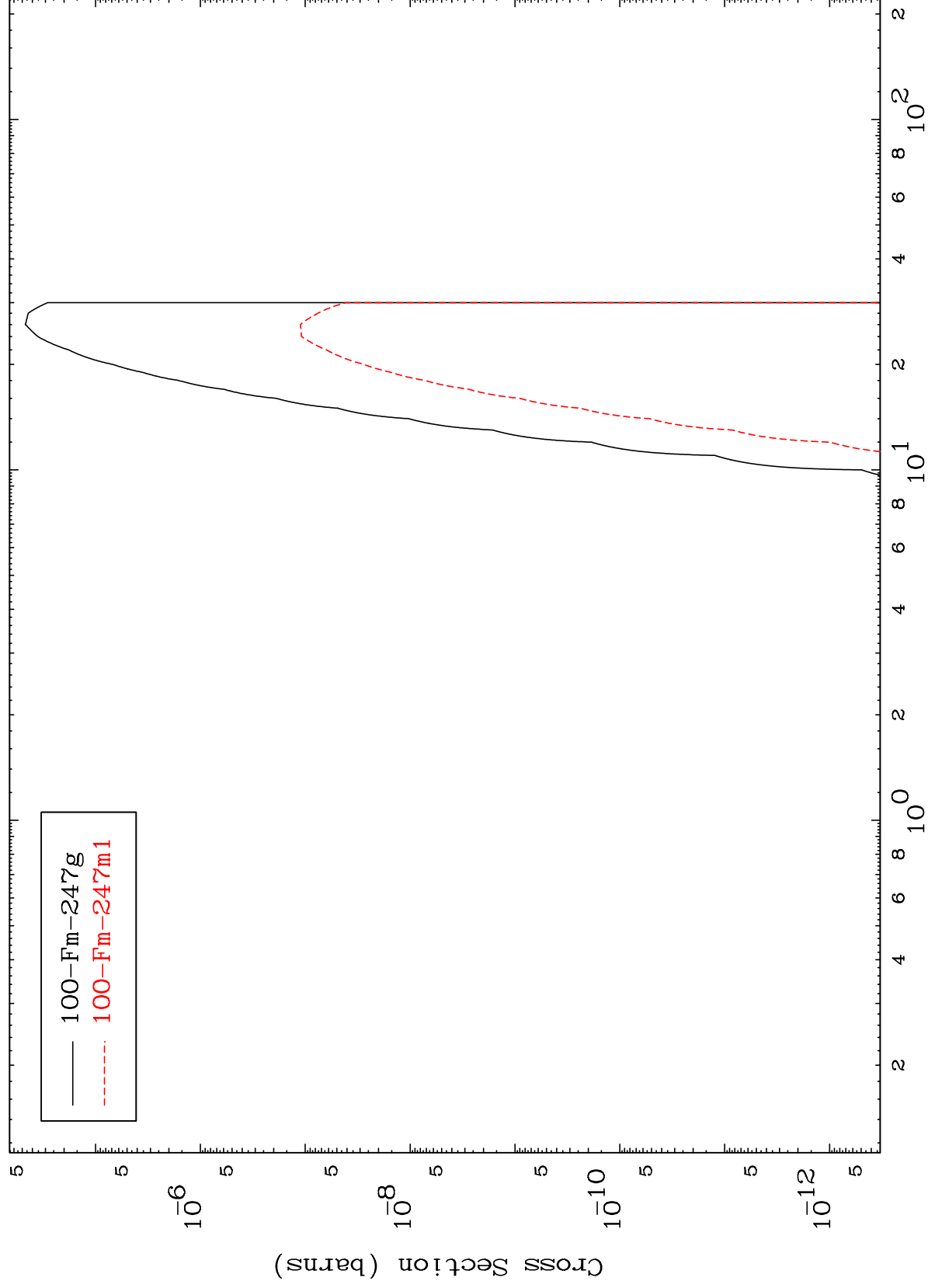


MAT 9951

(He-3,d) α

101-Md-250

Radionuclide Production Cross Section



16

Incident Energy (MeV)

101-Md-250