

Program EVALPLOT  
(Version 2018-1)

by

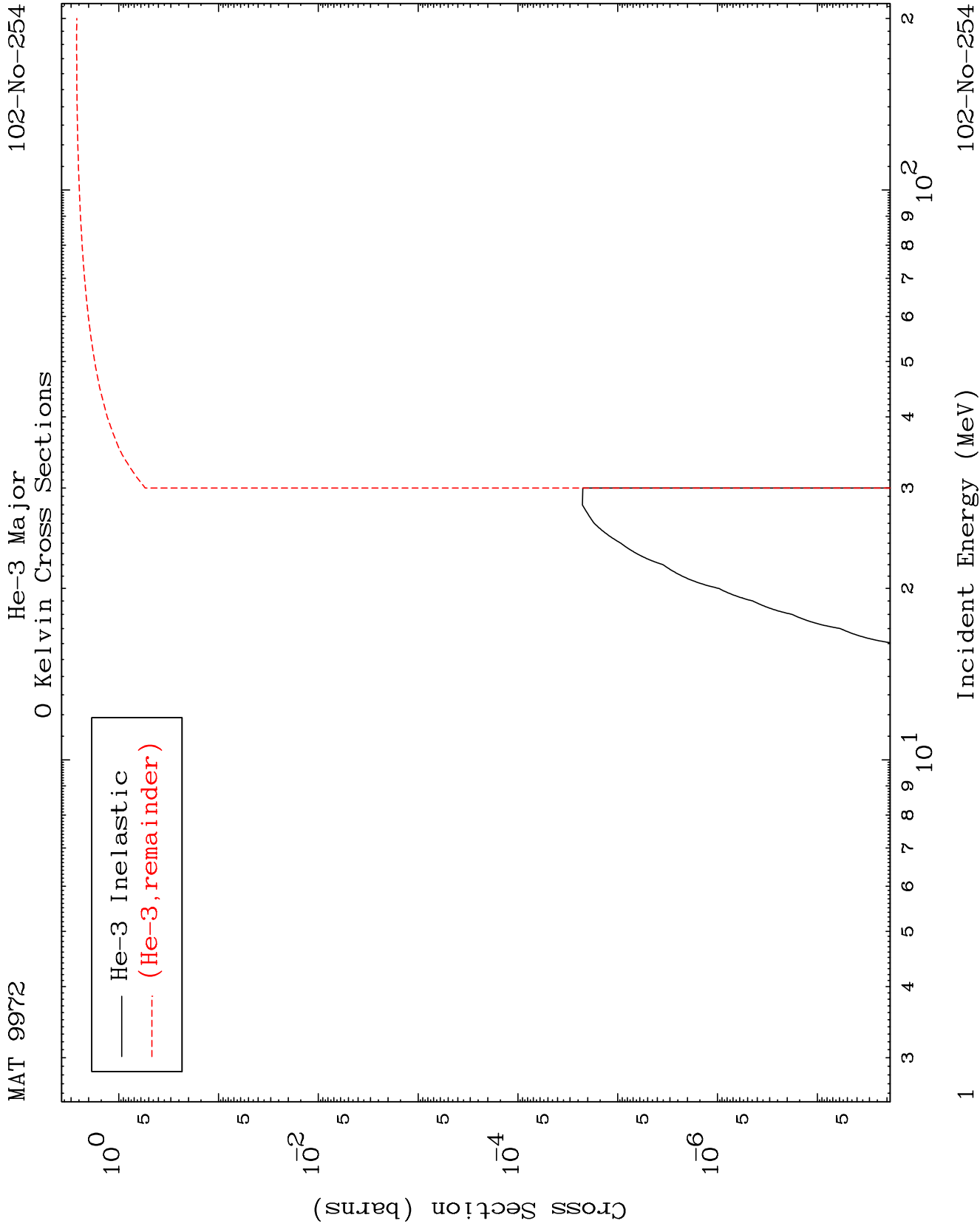
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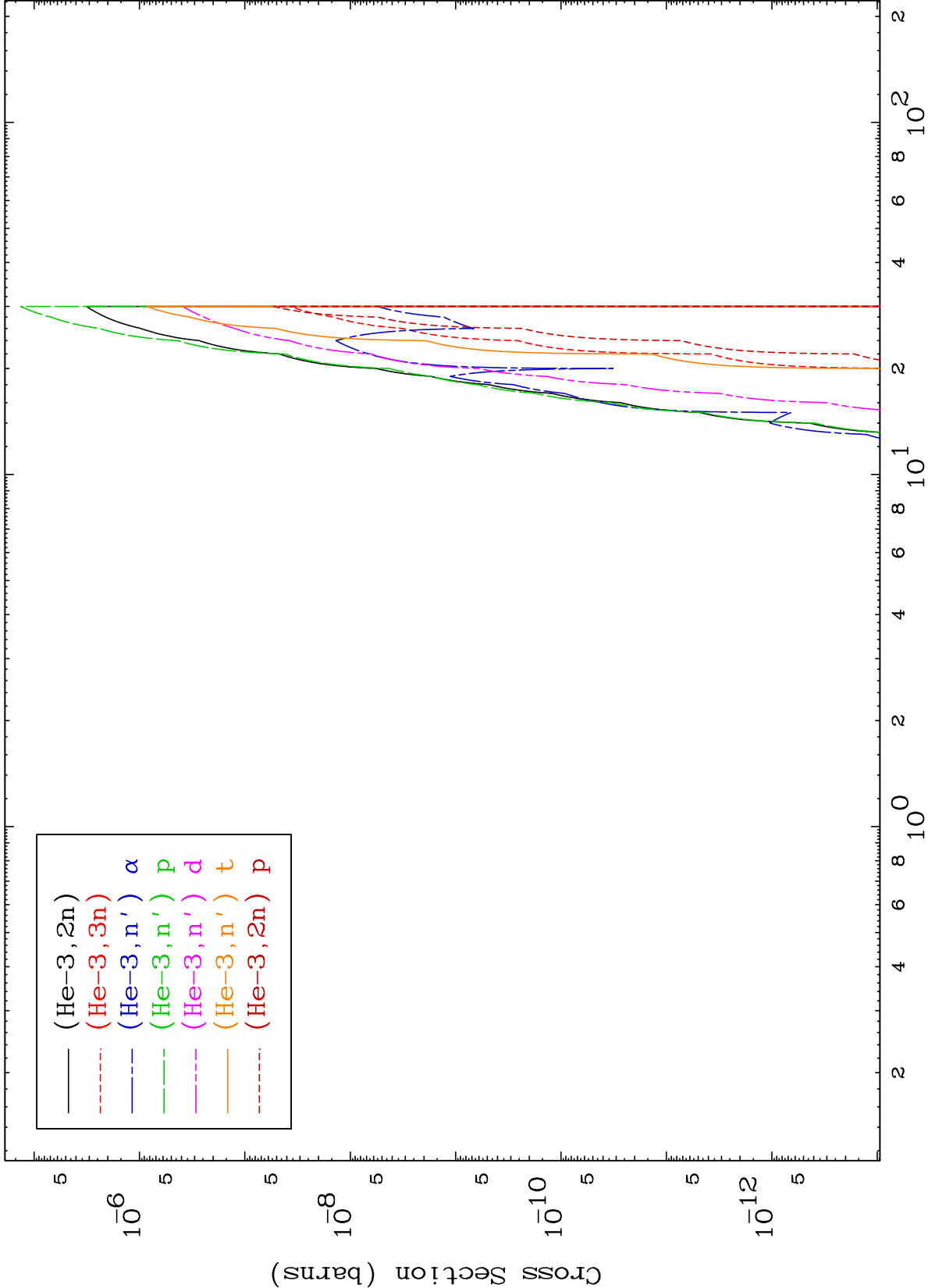
Press Mouse Button to Start

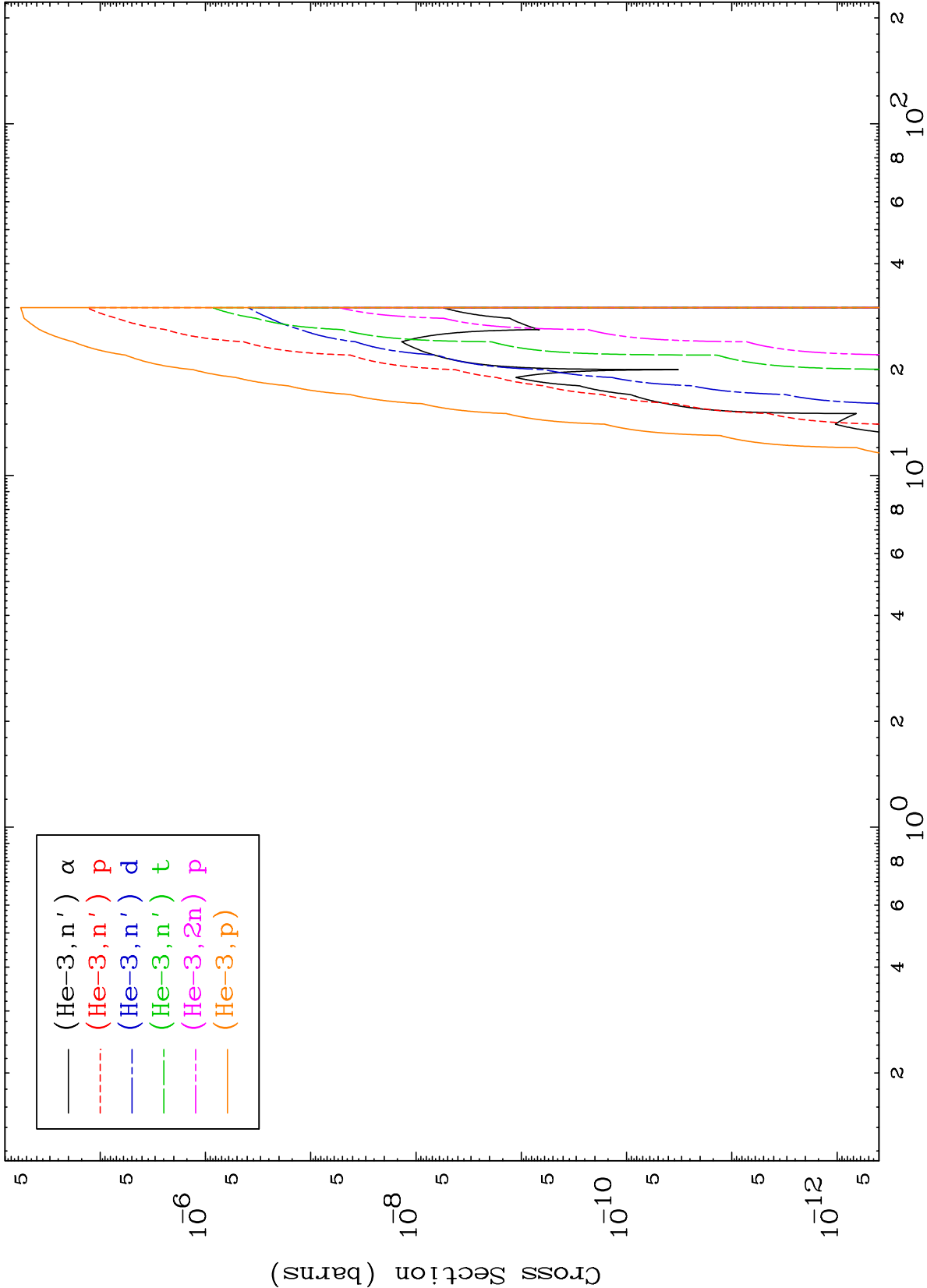


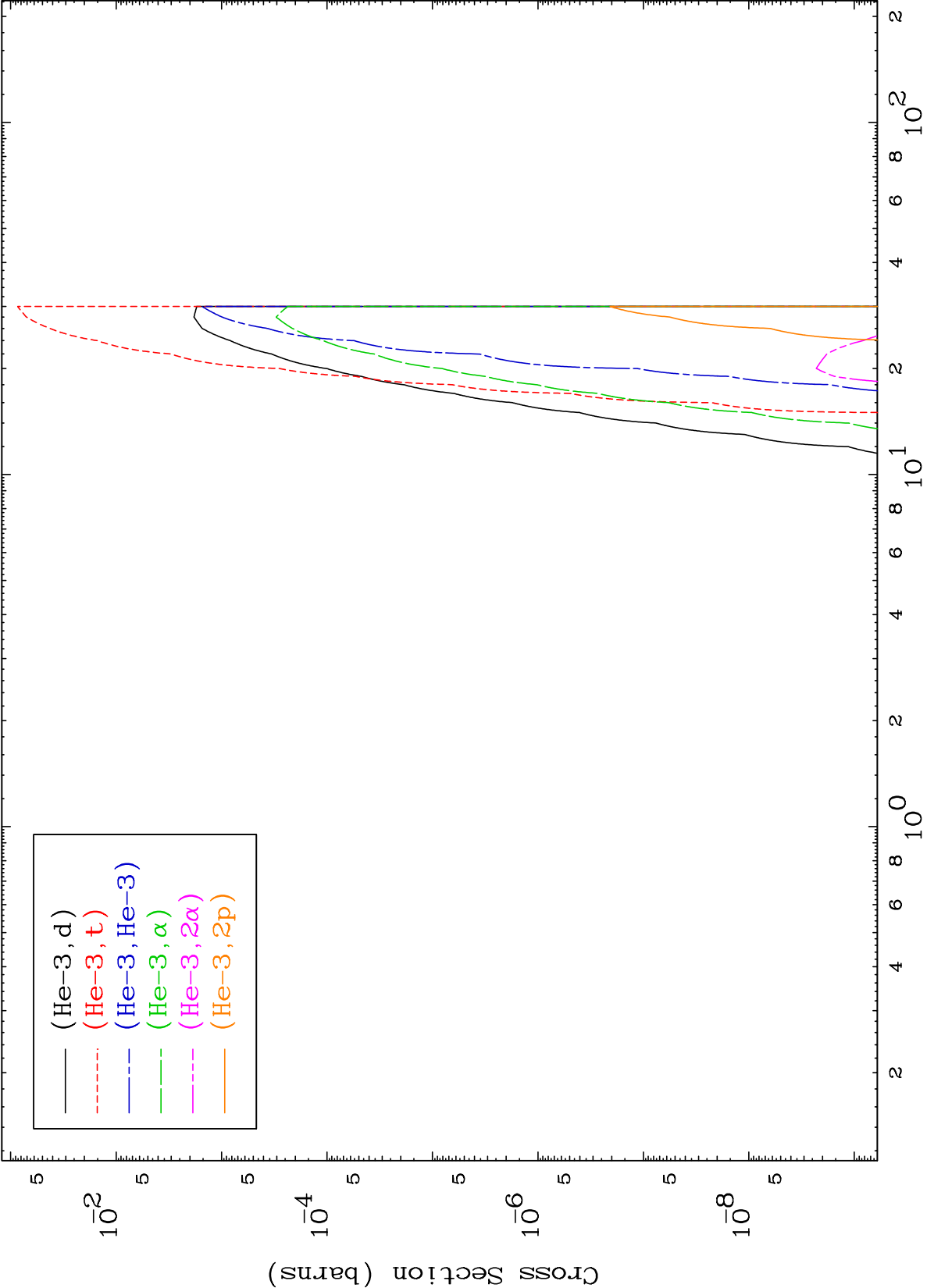
MAT 9972

He-3 Neutron Production  
0 Kelvin Cross Sections

102-No-254



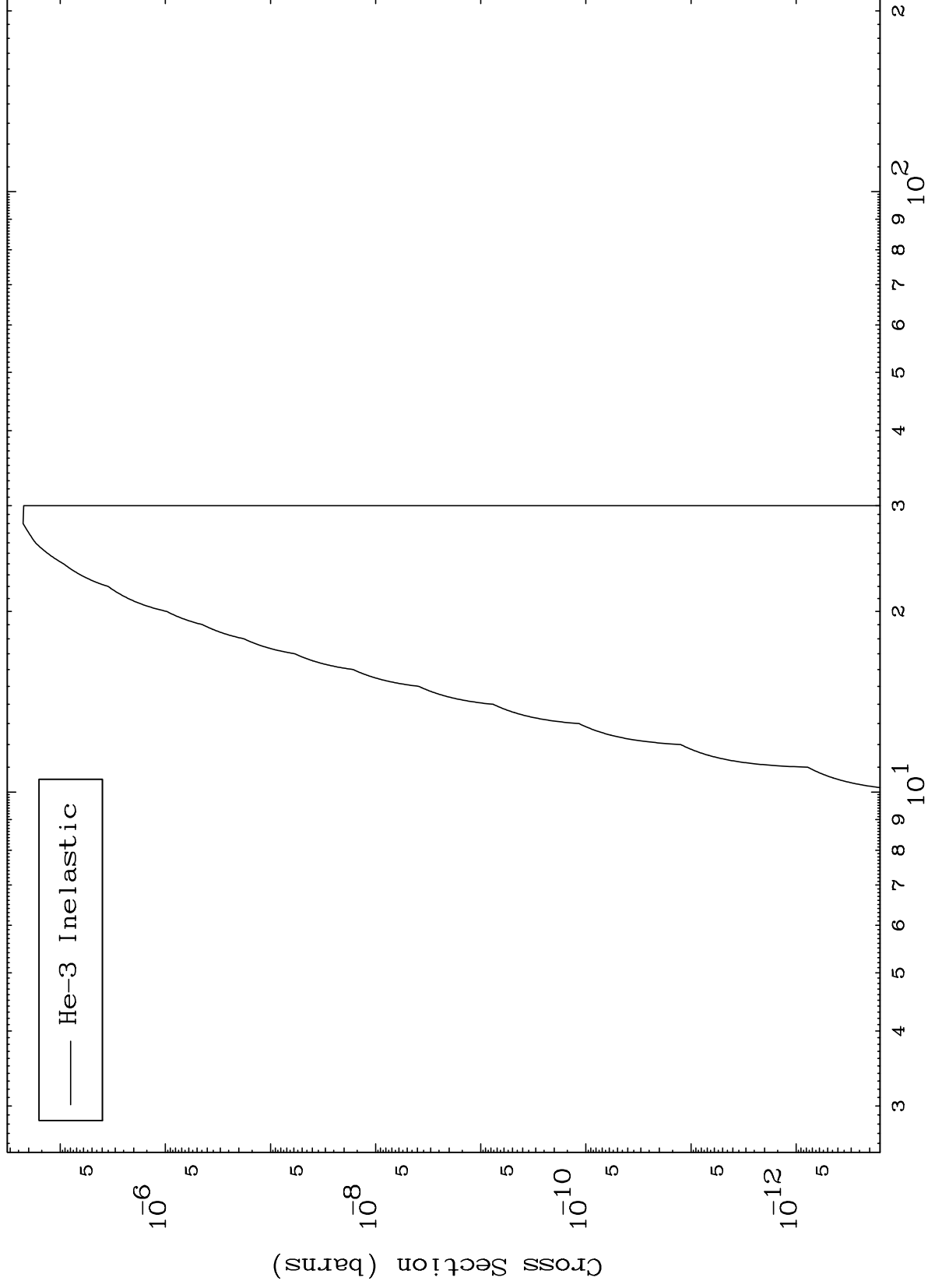




MAT 9972

102-No-254

(He-3,n') Level  
0 Kelvin Cross Sections

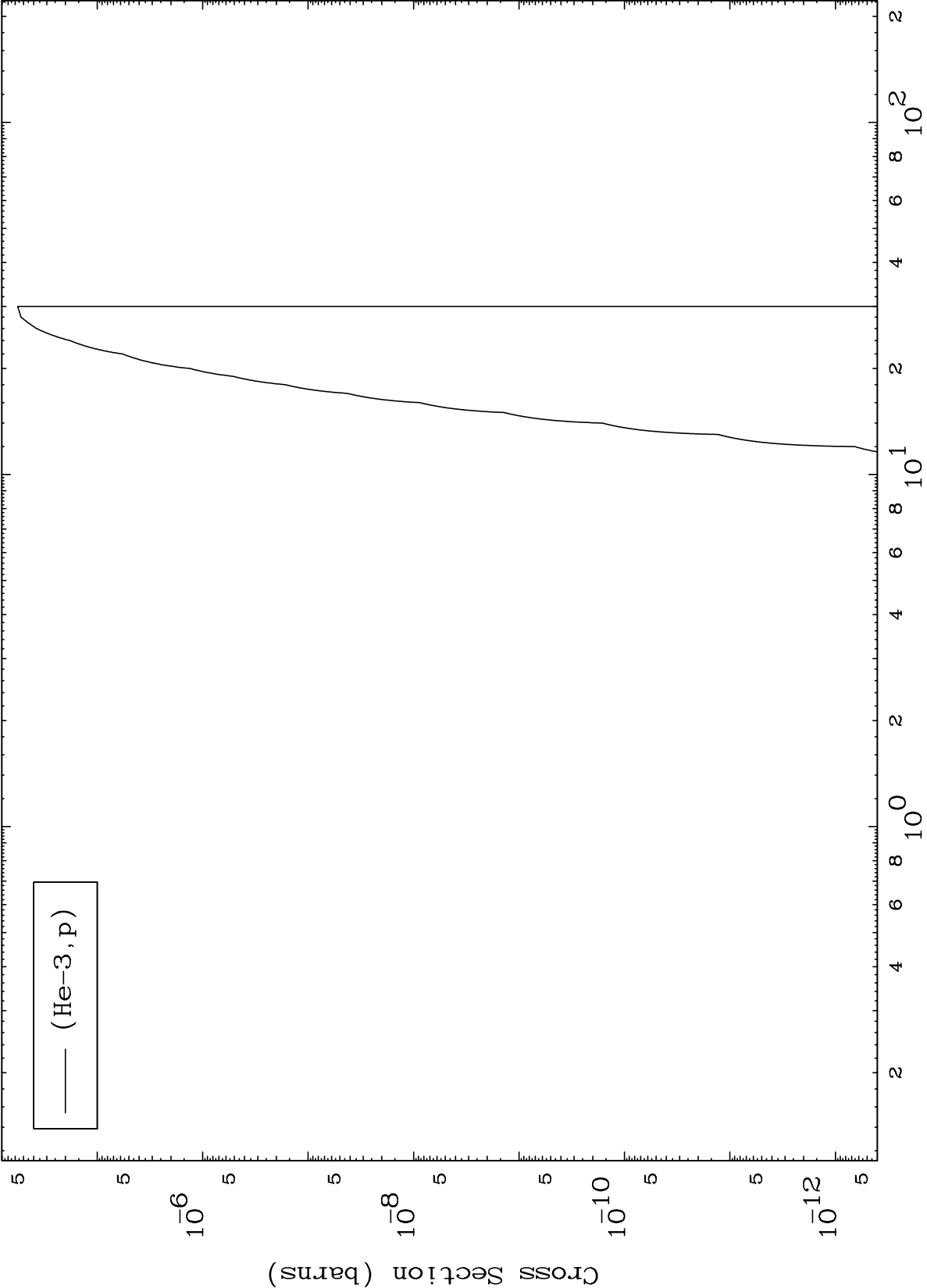


102-No-254

Incident Energy (MeV)

5

0 Kelvin Cross Sections

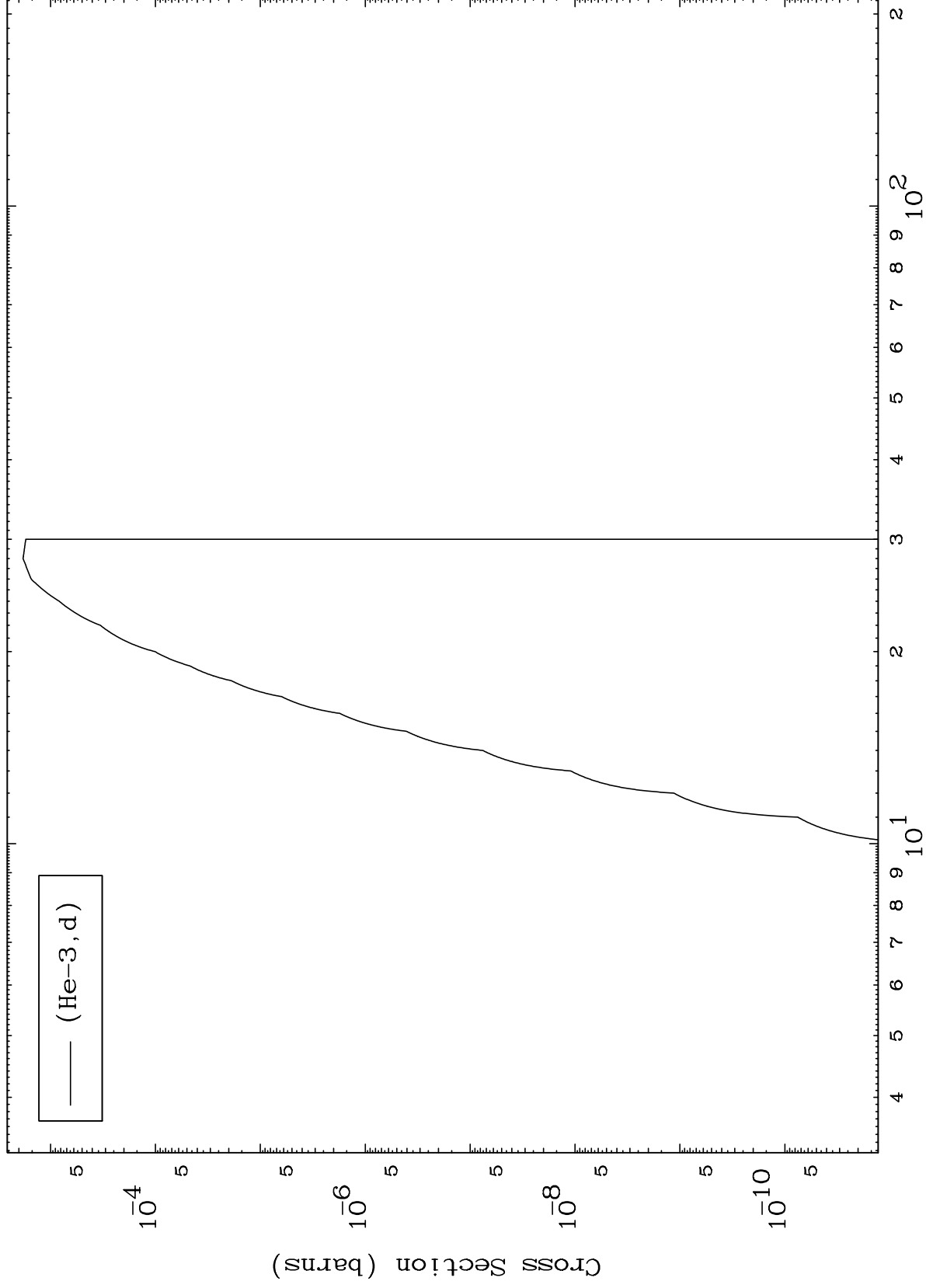


— (He-3, p)

MAT 9972

102-No-254

(He-3,d) Levels  
0 Kelvin Cross Sections



102-No-254

Incident Energy (MeV)

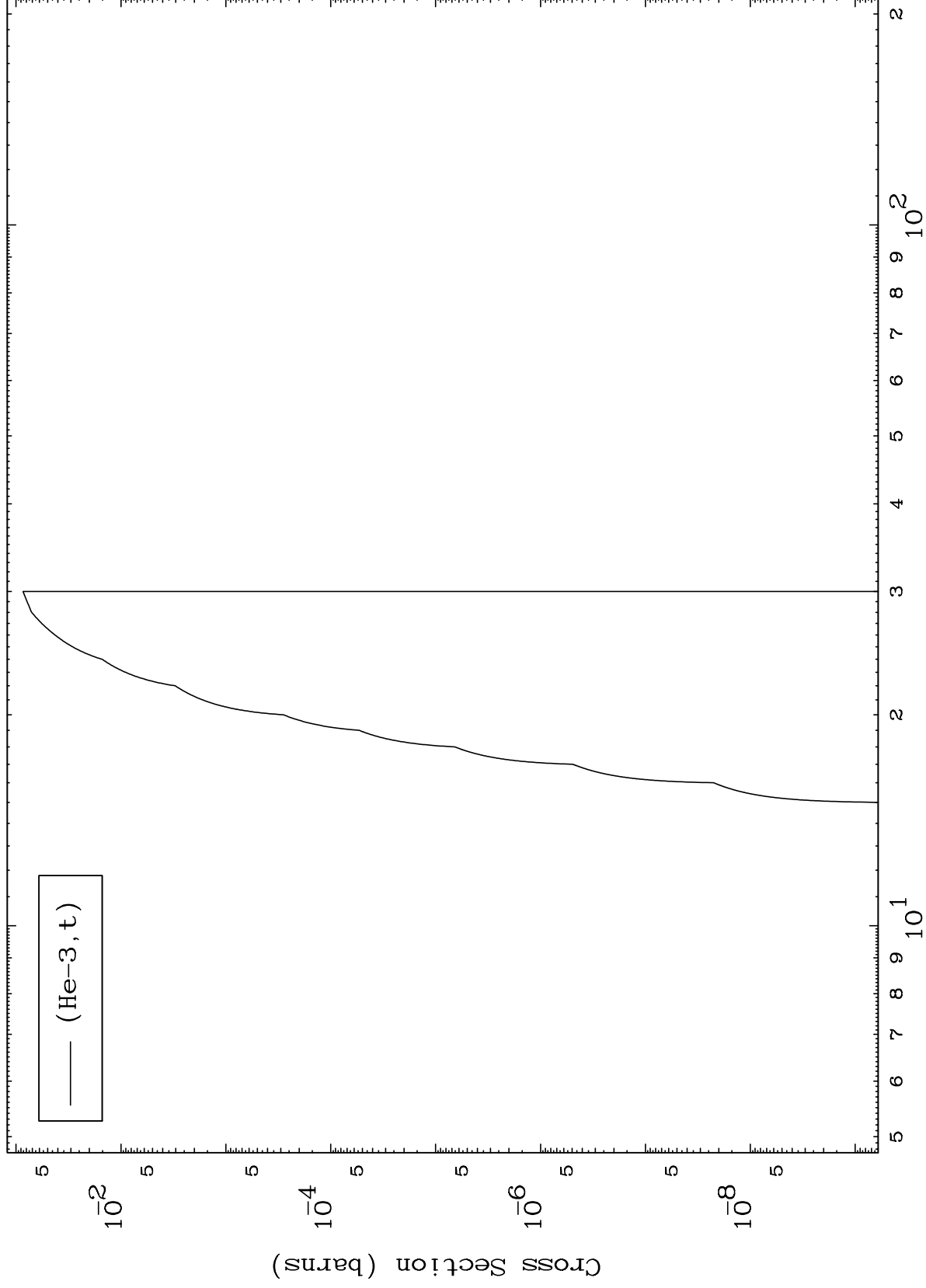
7



MAT 9972

102-No-254

(He-3,t) Levels  
0 Kelvin Cross Sections

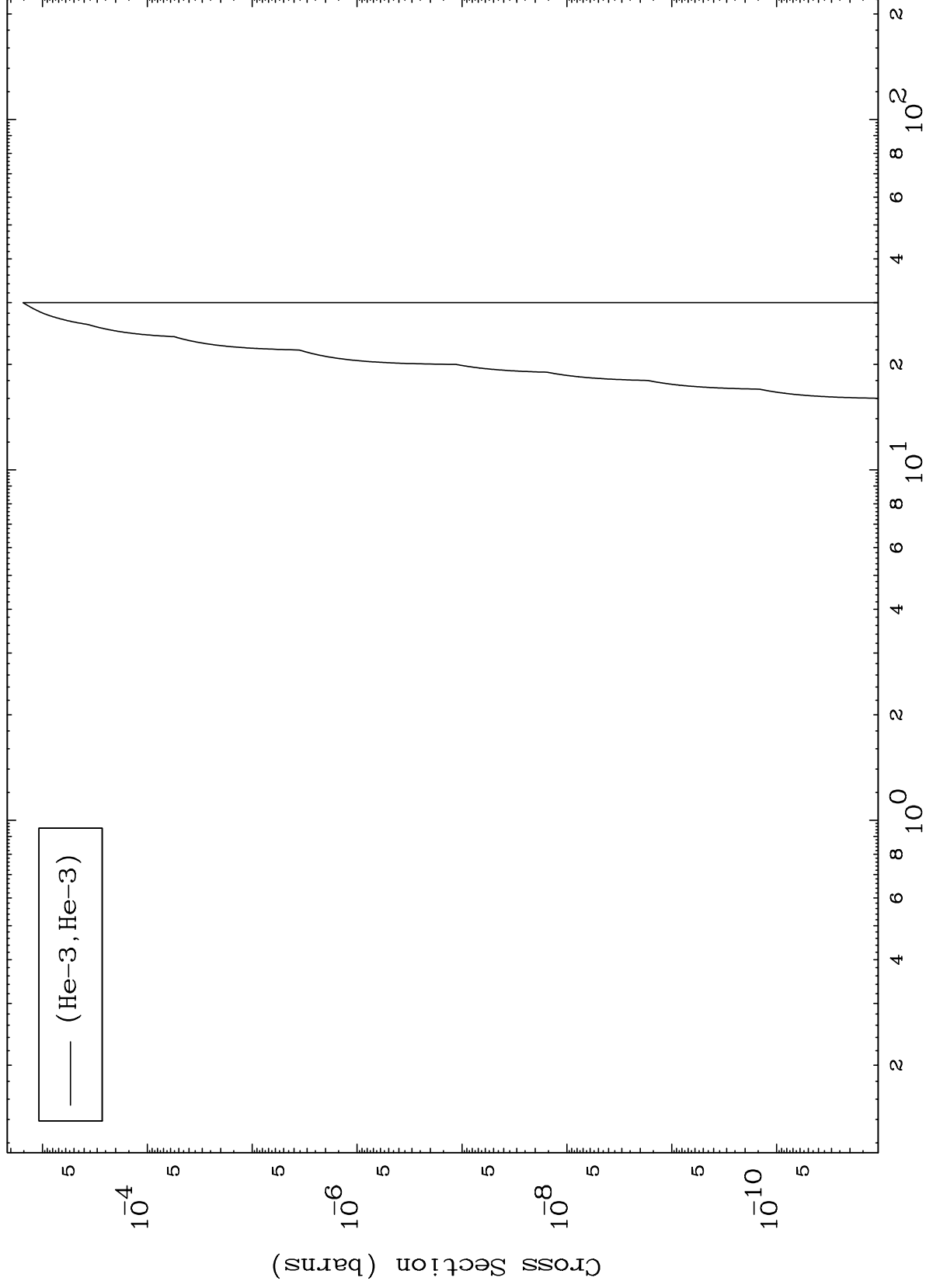


8

Incident Energy (MeV)

102-No-254

(He-3, He3) Levels  
0 Kelvin Cross Sections

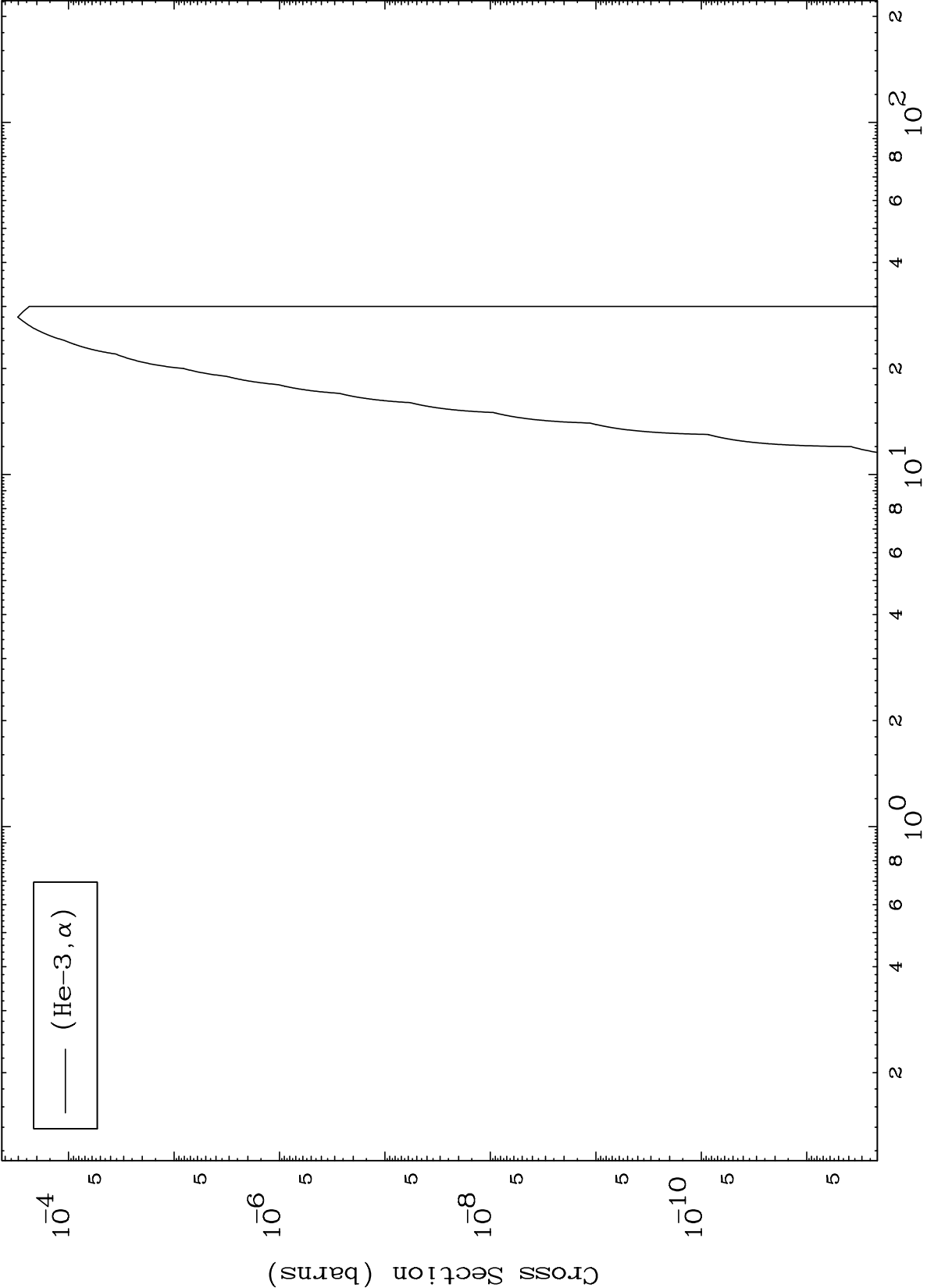


MAT 9972

(He-3,  $\alpha$ ) Levels

102-No-254

0 Kelvin Cross Sections



10

Incident Energy (MeV)

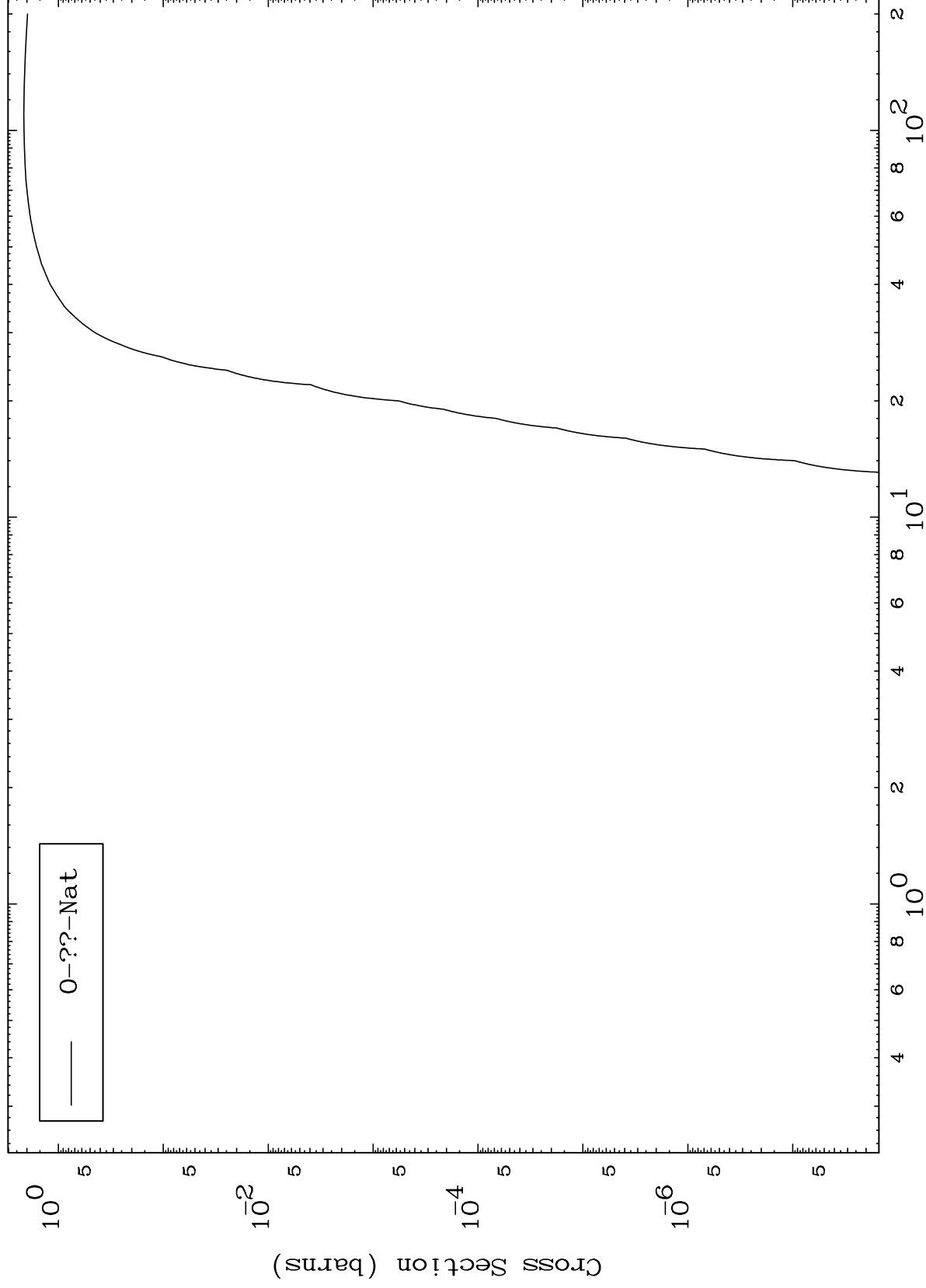
102-No-254

MAT 9972

He-3 Fission

102-No-254

Radionuclide Production Cross Section

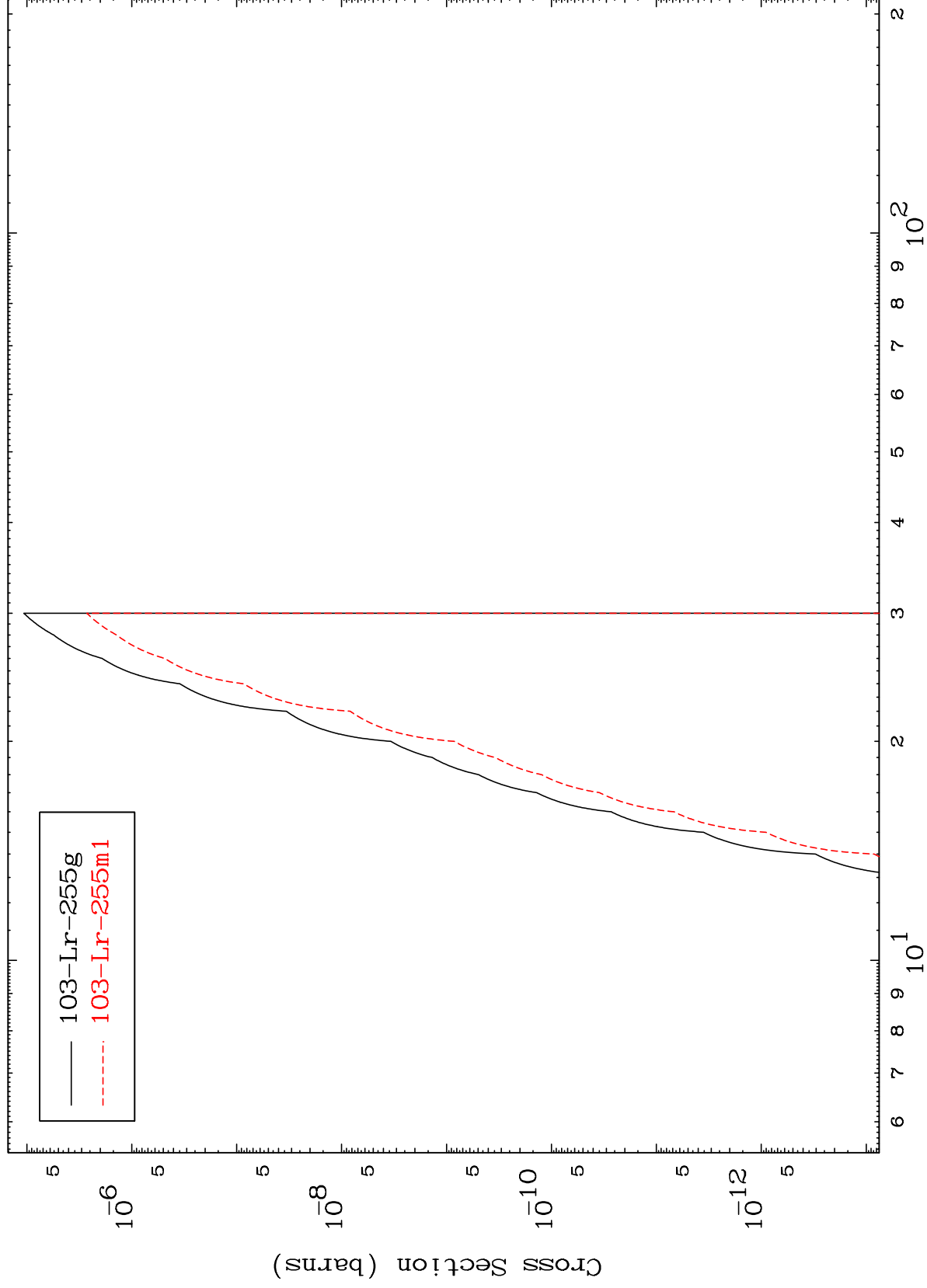


0-??-Nat

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102-No-254

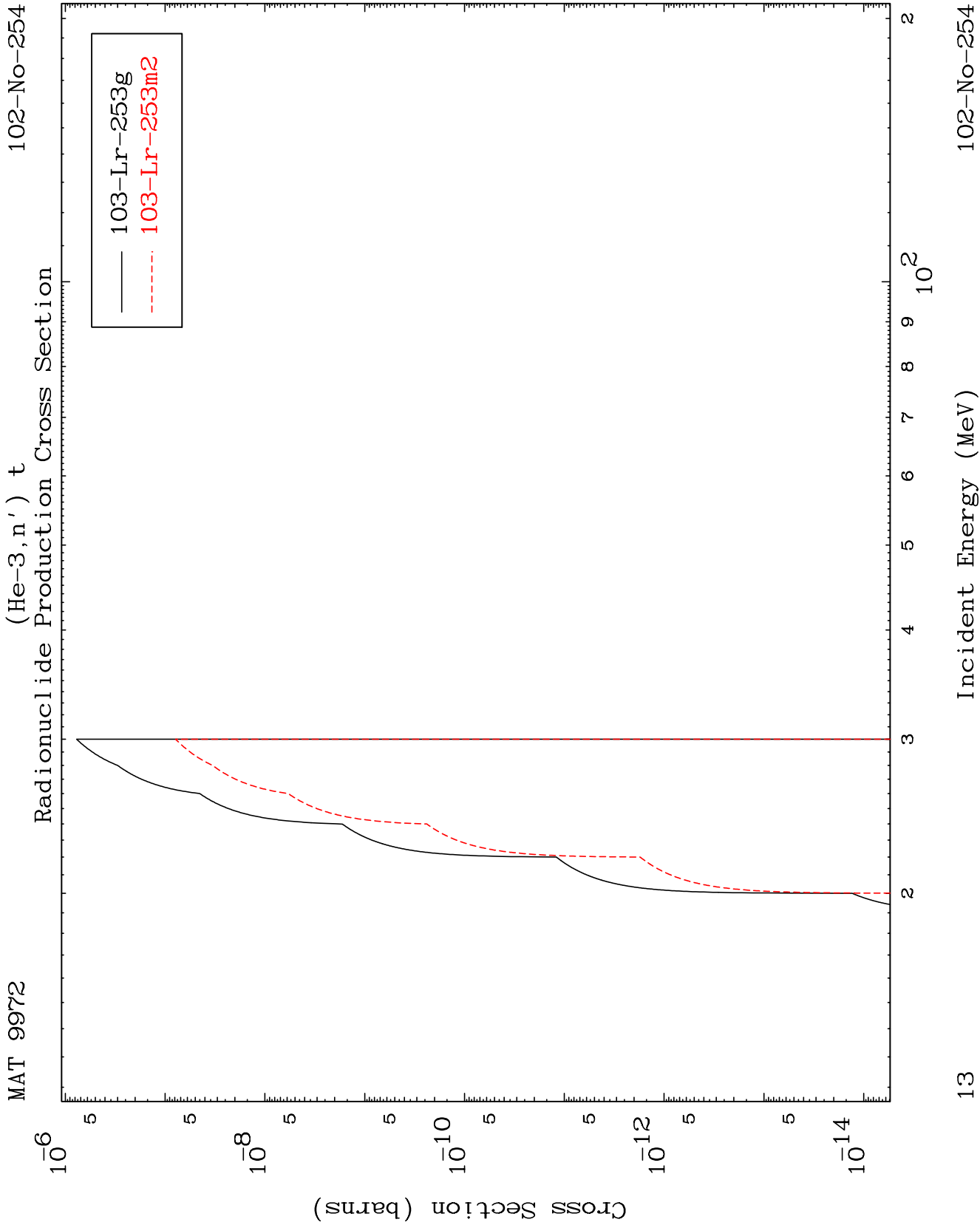
(He-3,n') p  
Radionuclide Production Cross Section



102-No-254

Incident Energy (MeV)

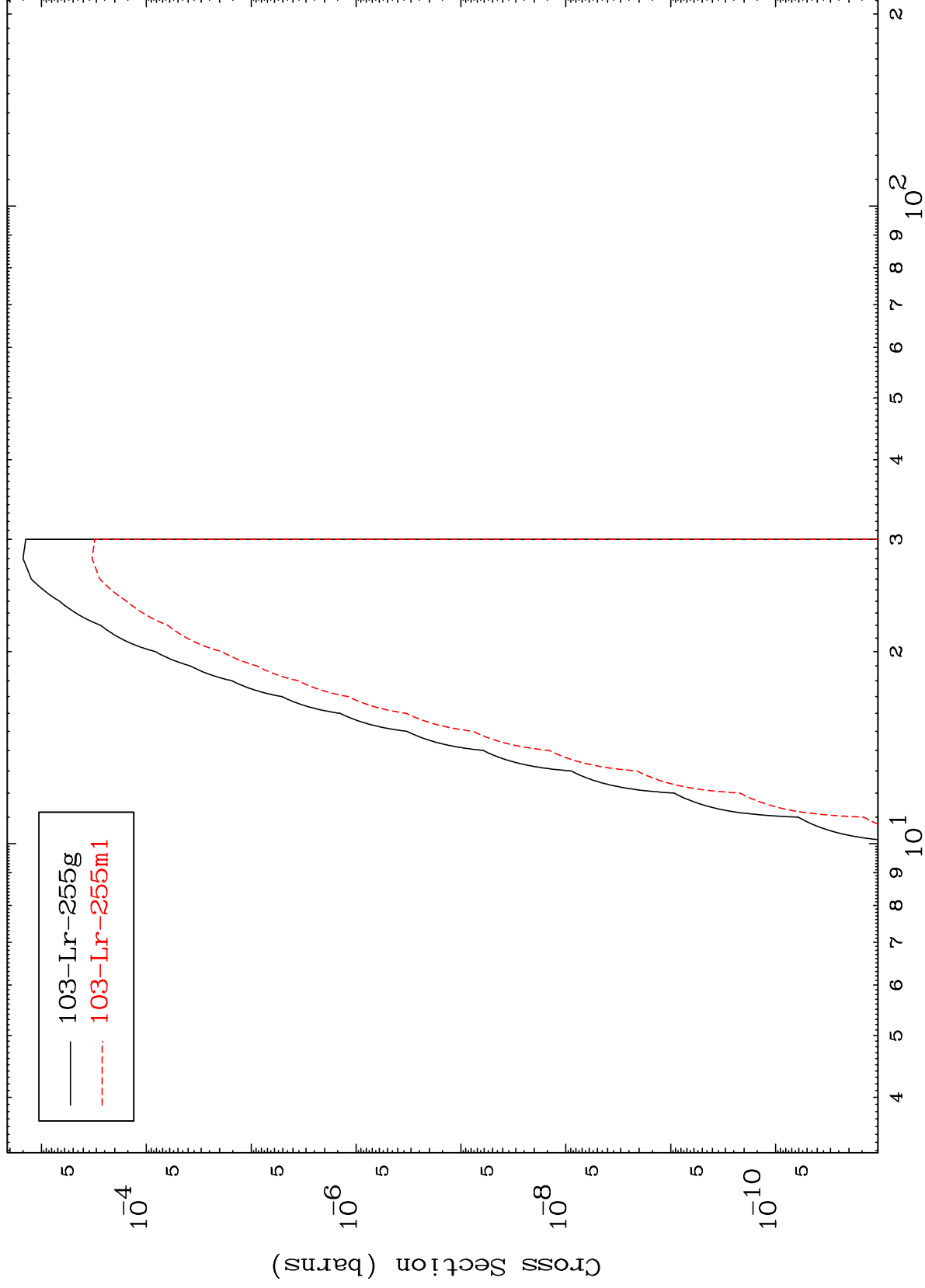
12



MAT 9972

102-No-254

Radionuclide Production Cross Section  
(He-3,d)



102-No-254

Incident Energy (MeV)

14