

Program EVALPLOT
(Version 2018-1)

by

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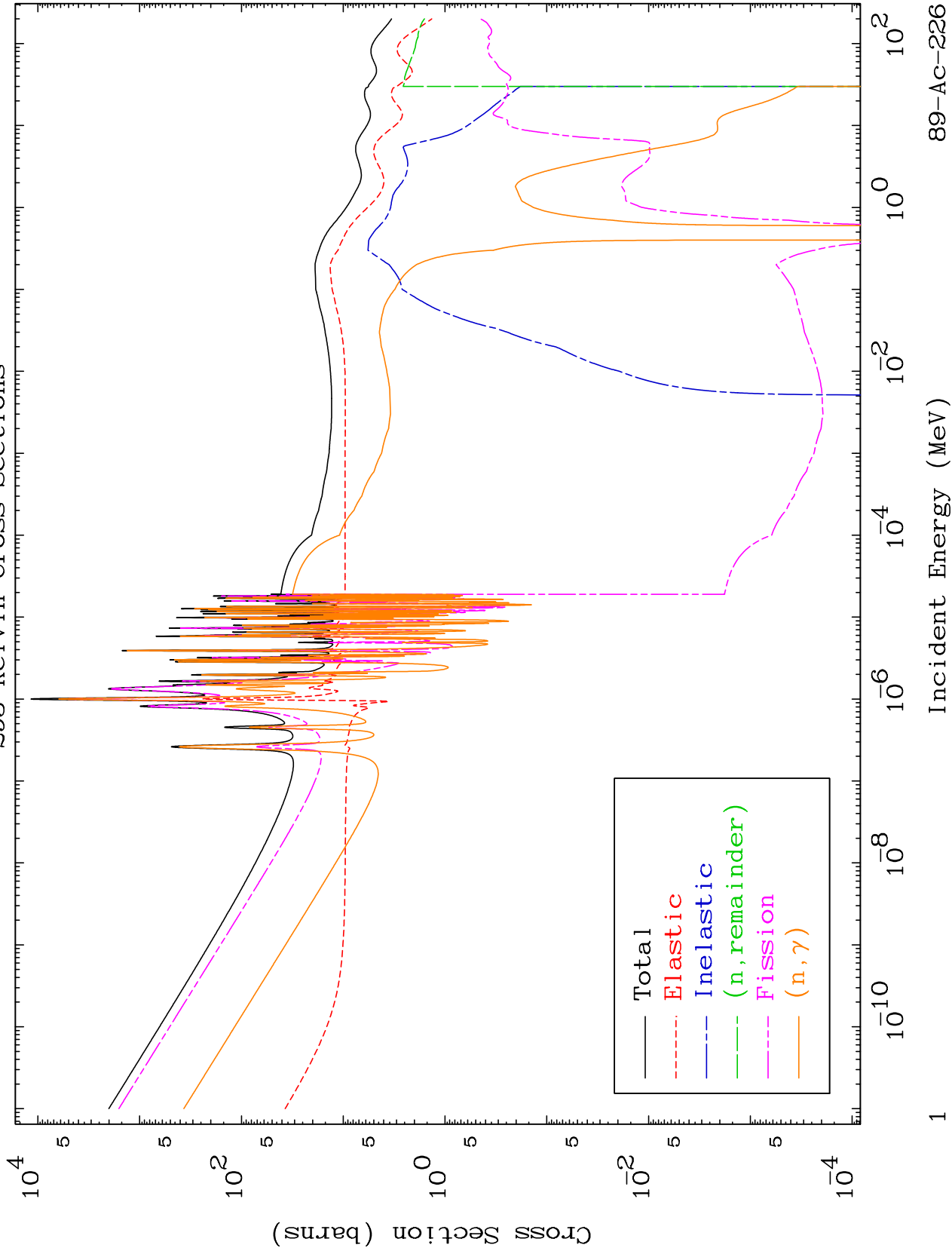
Press Mouse Button to Start

MAT 8928

Major

293 Kelvin Cross Sections

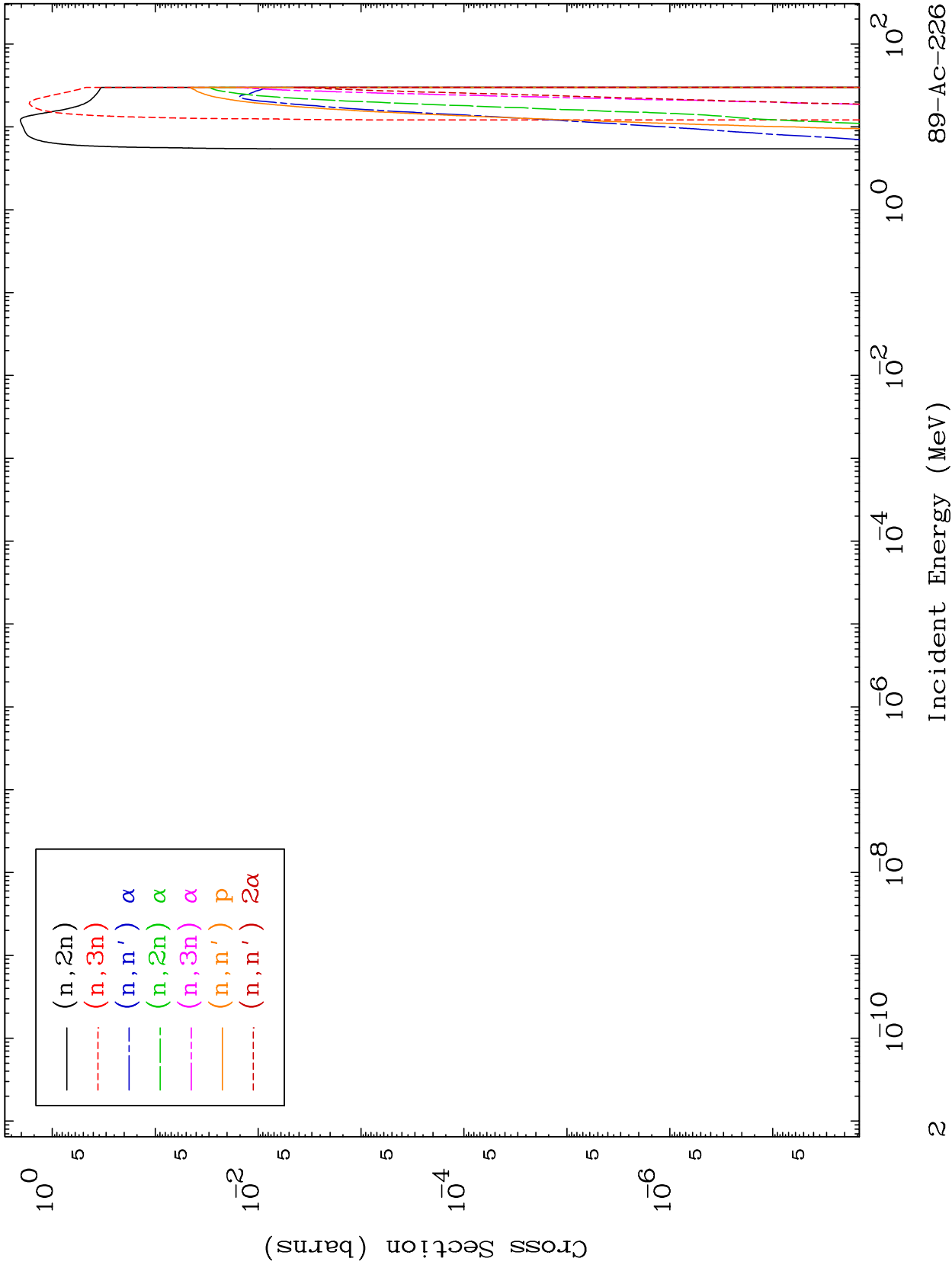
89-Ac-226



MAT 8928

Neutron Production
293 Kelvin Cross Sections

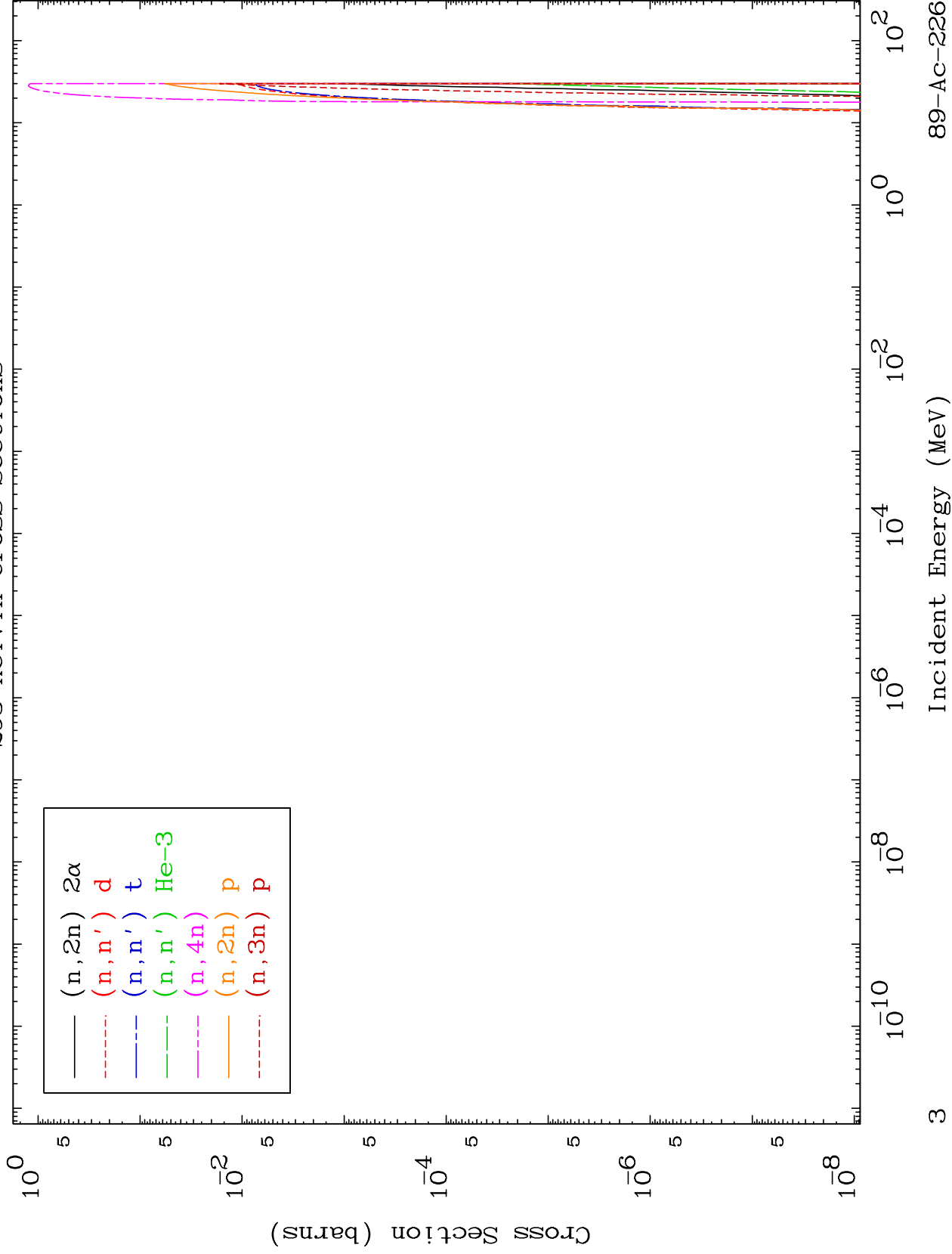
89-Ac-226



MAT 8928

Neutron Production
293 Kelvin Cross Sections

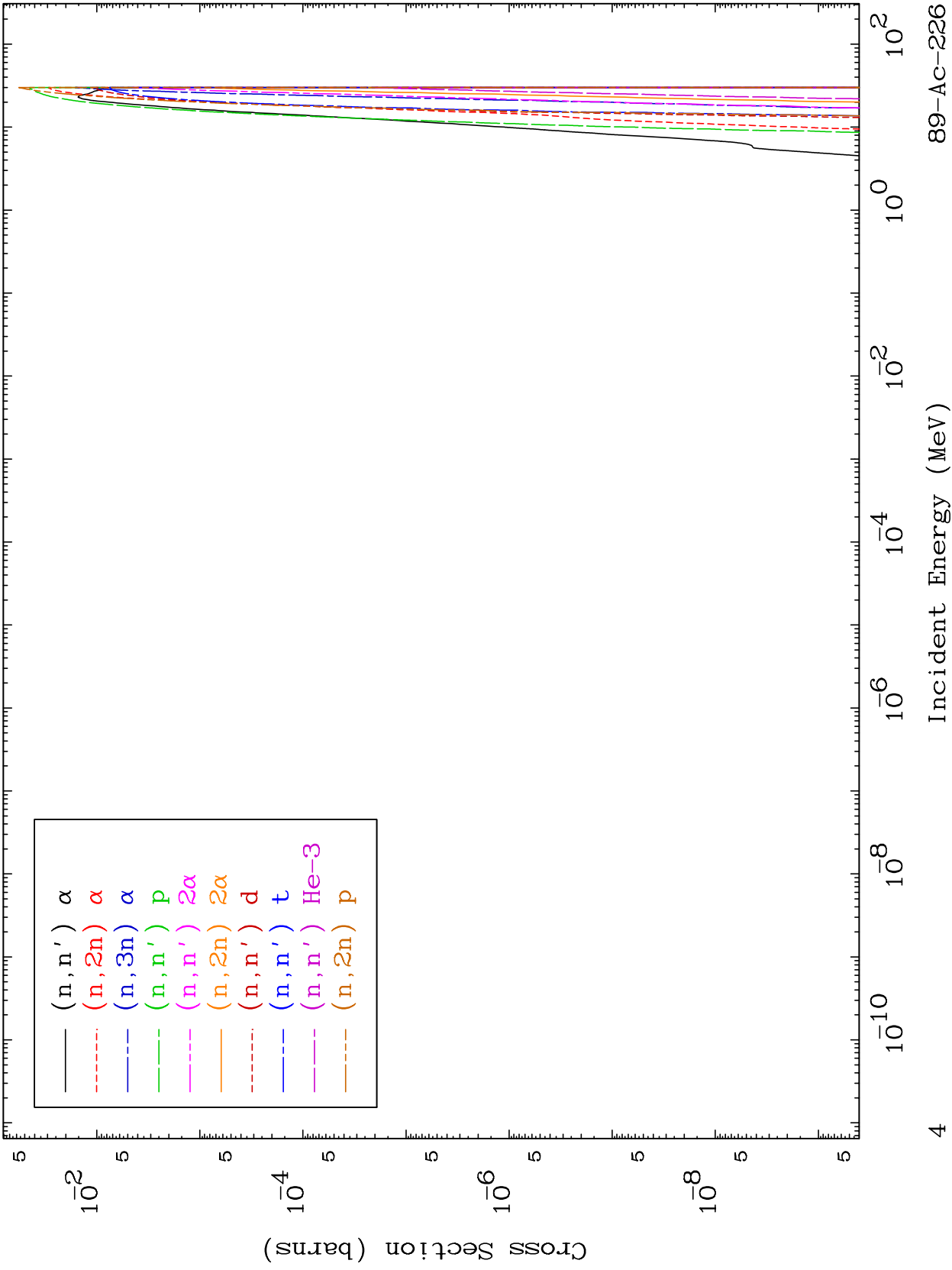
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MAT 8928

Charged Particle
293 Kelvin Cross Sections

89-Ac-226

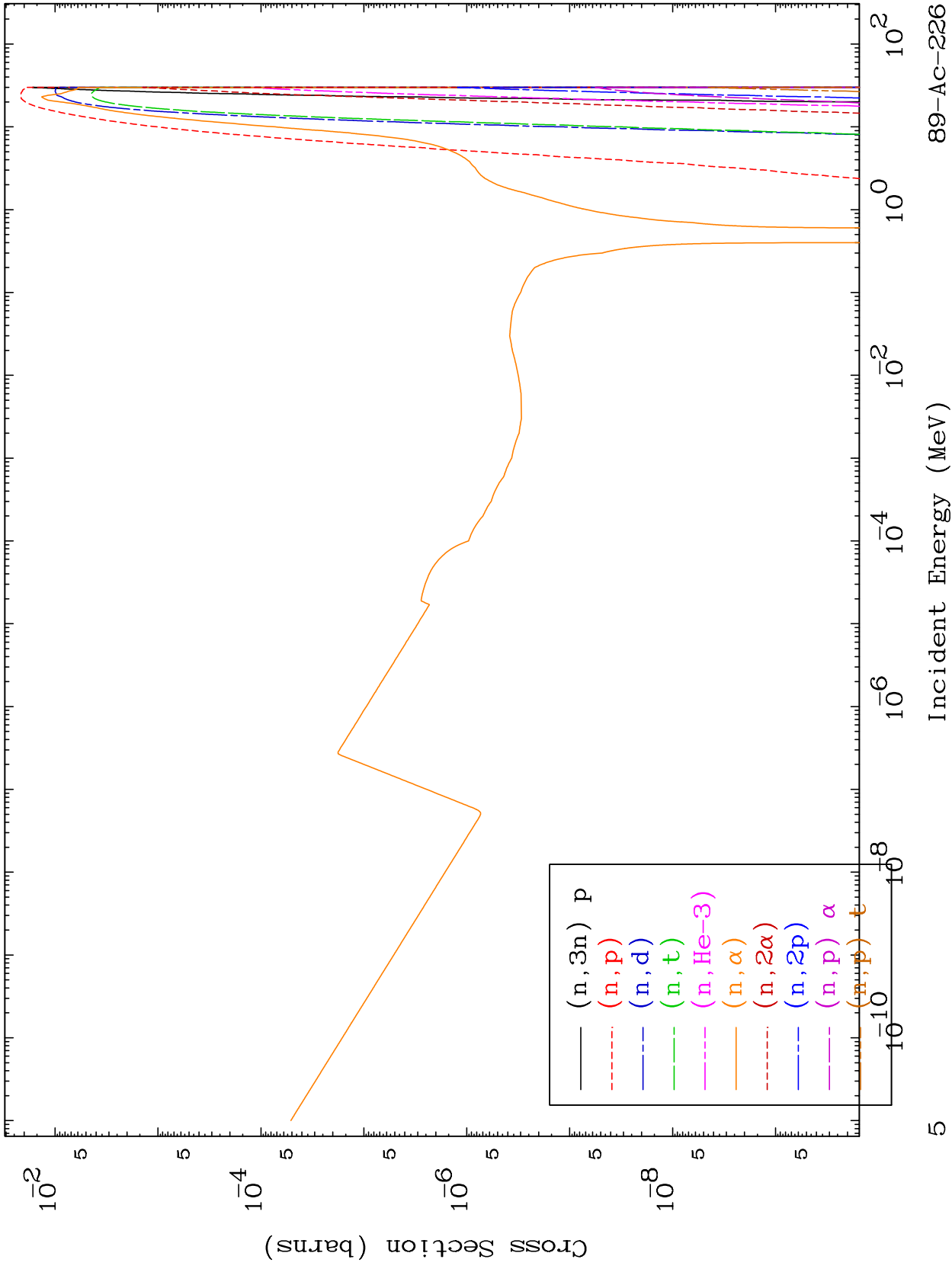


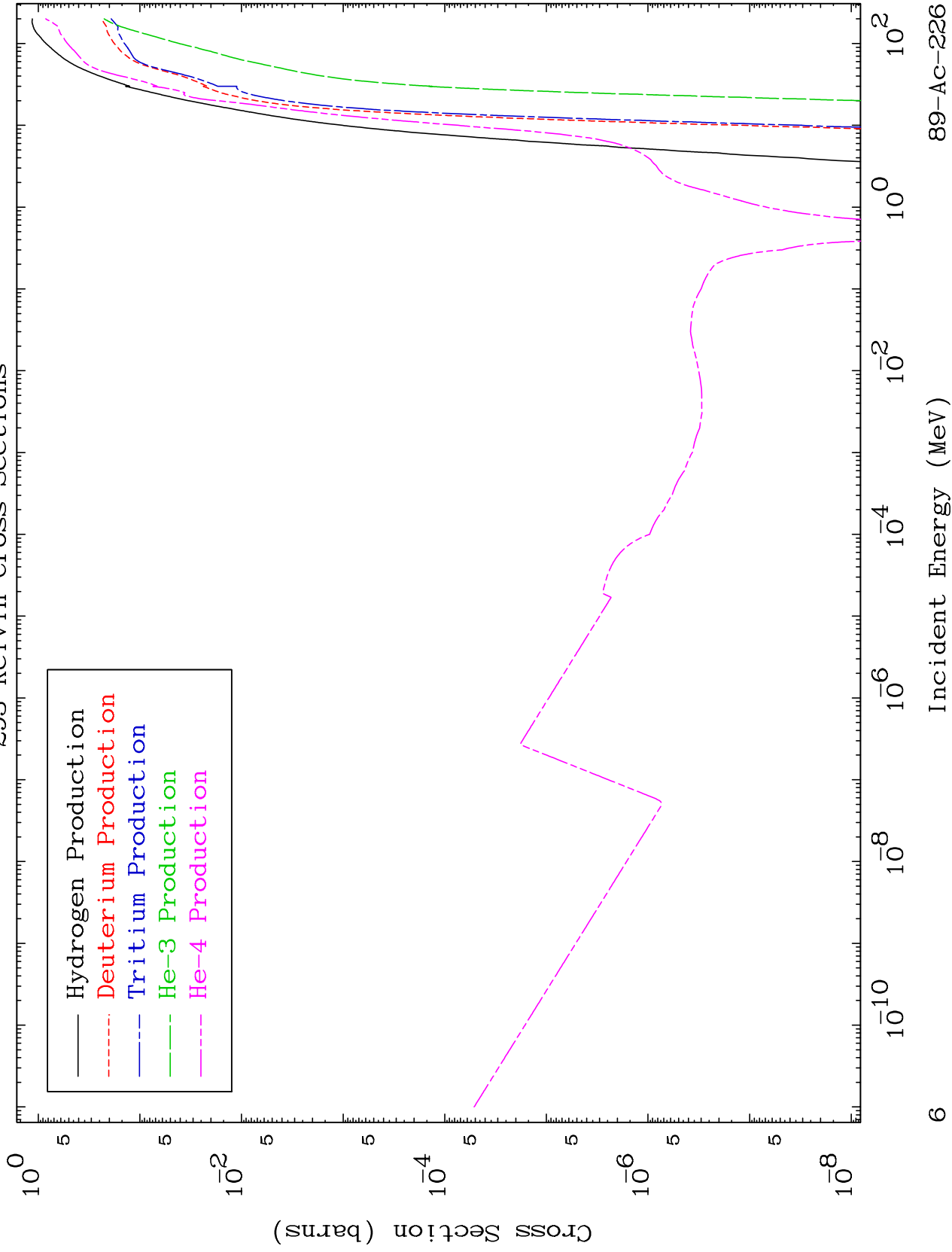
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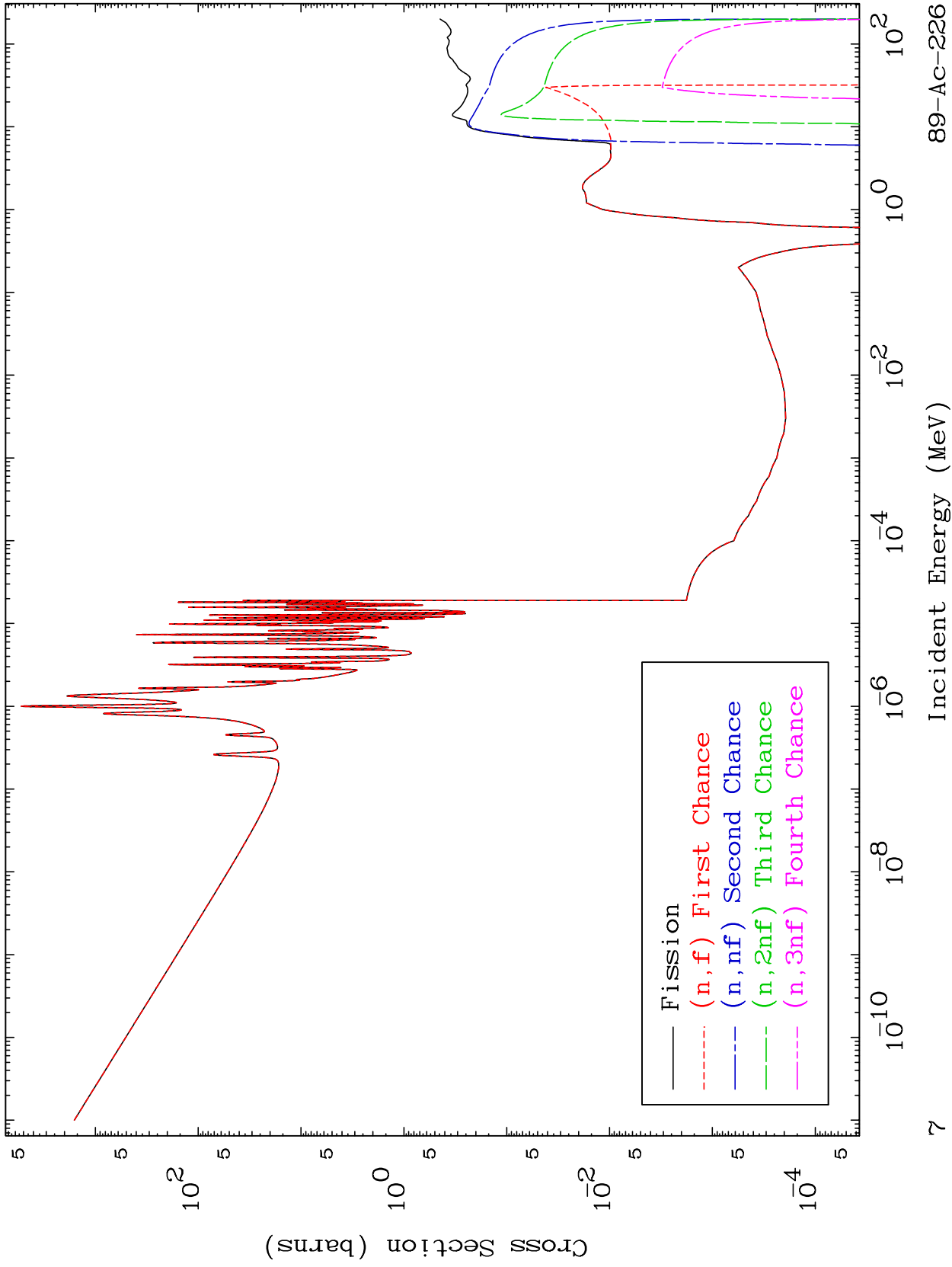
MAT 8928

Charged Particle
293 Kelvin Cross Sections

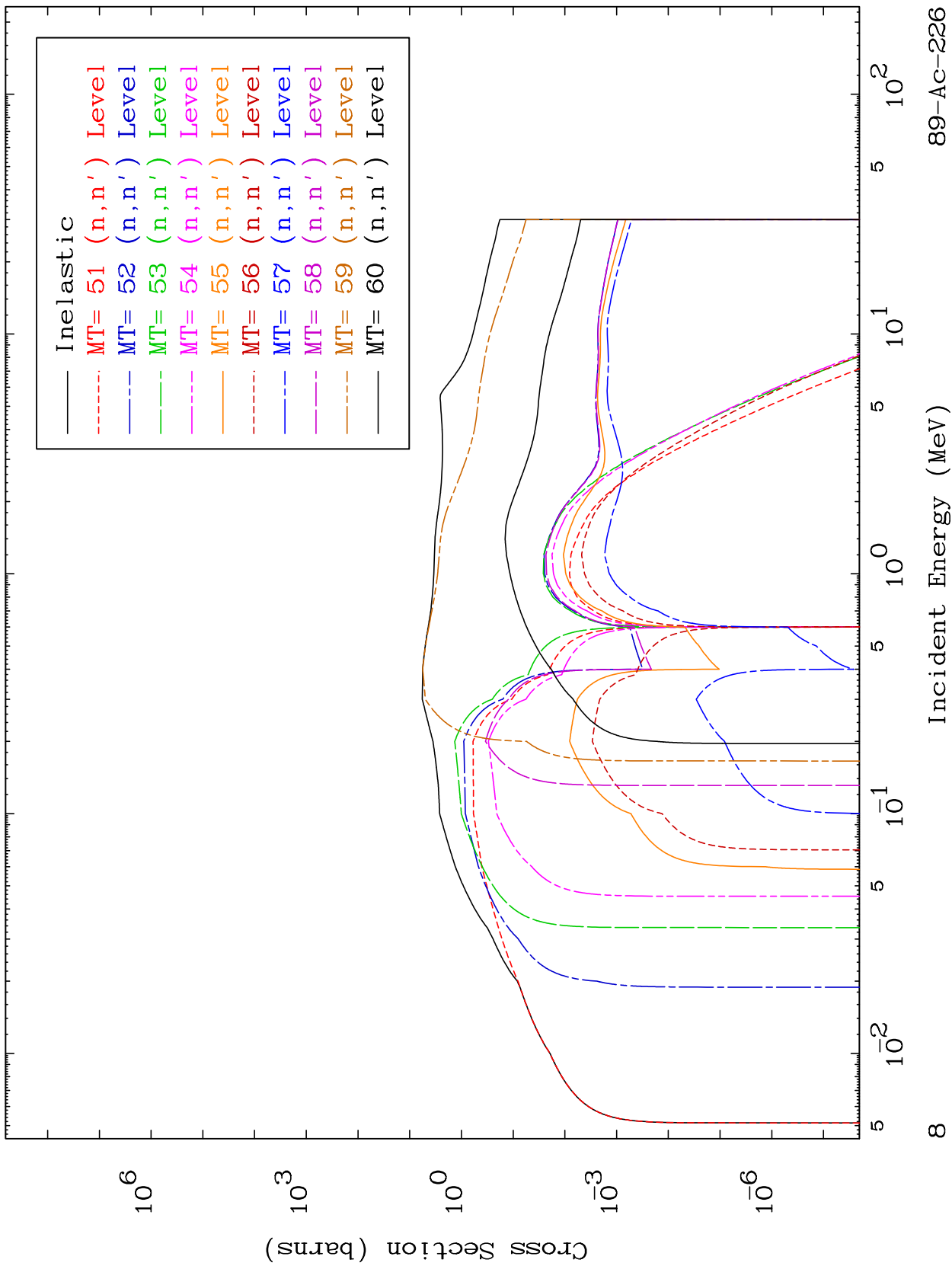
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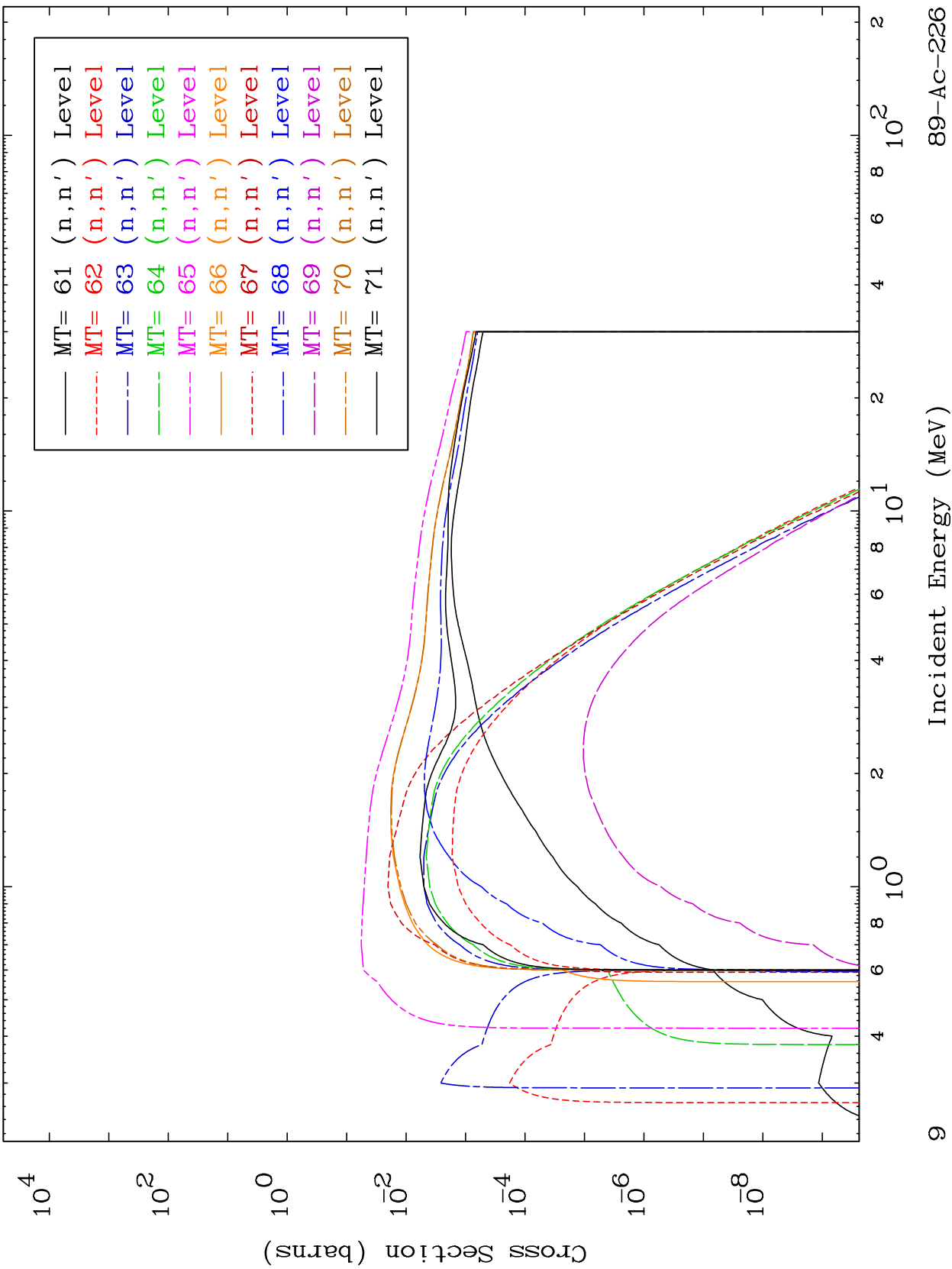






293 Kelvin Cross Sections



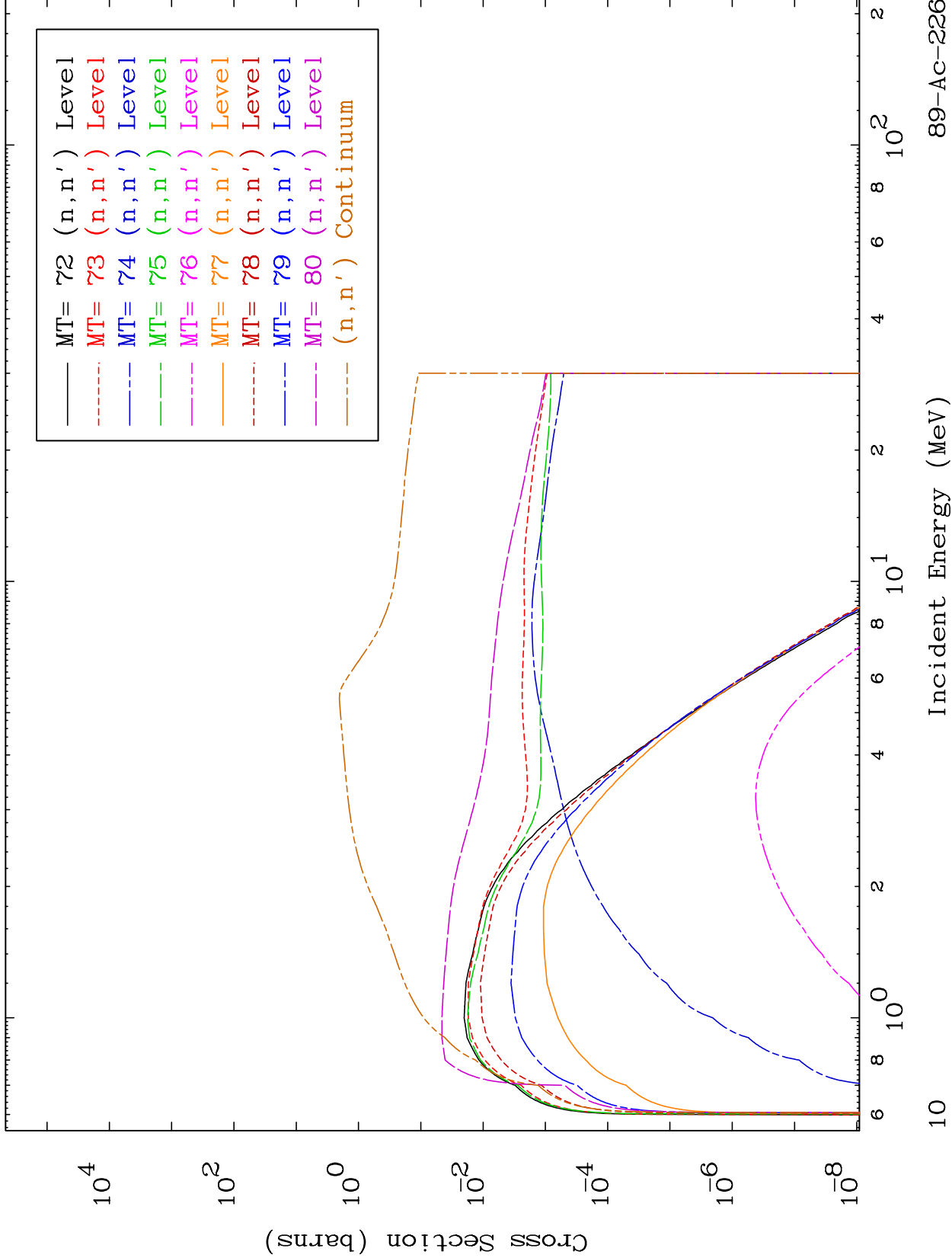


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(n,n') Level

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293 Kelvin Cross Sections



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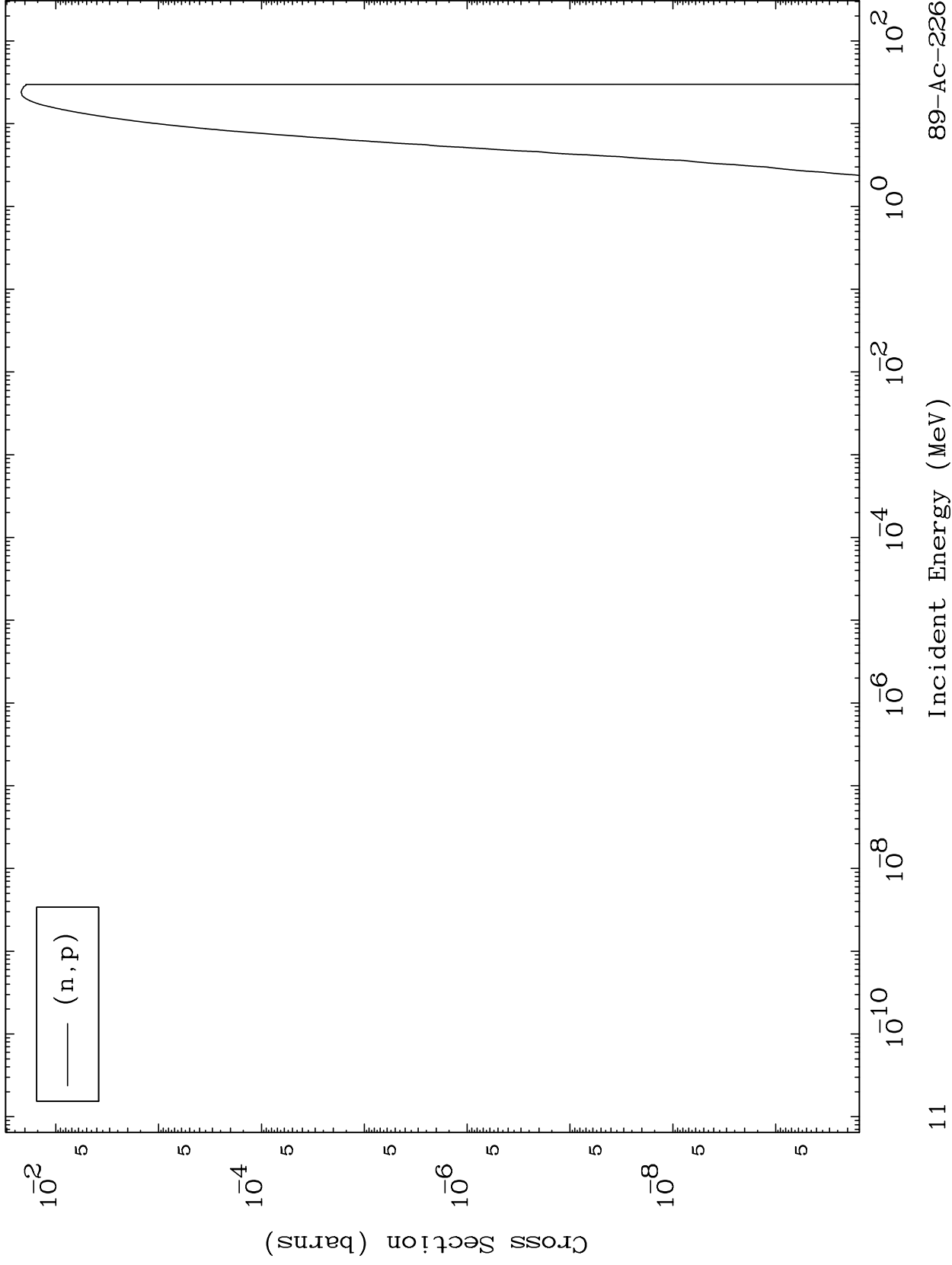
Incident Energy (MeV)

10

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(n,p) Levels
293 Kelvin Cross Sections

89-Ac-226

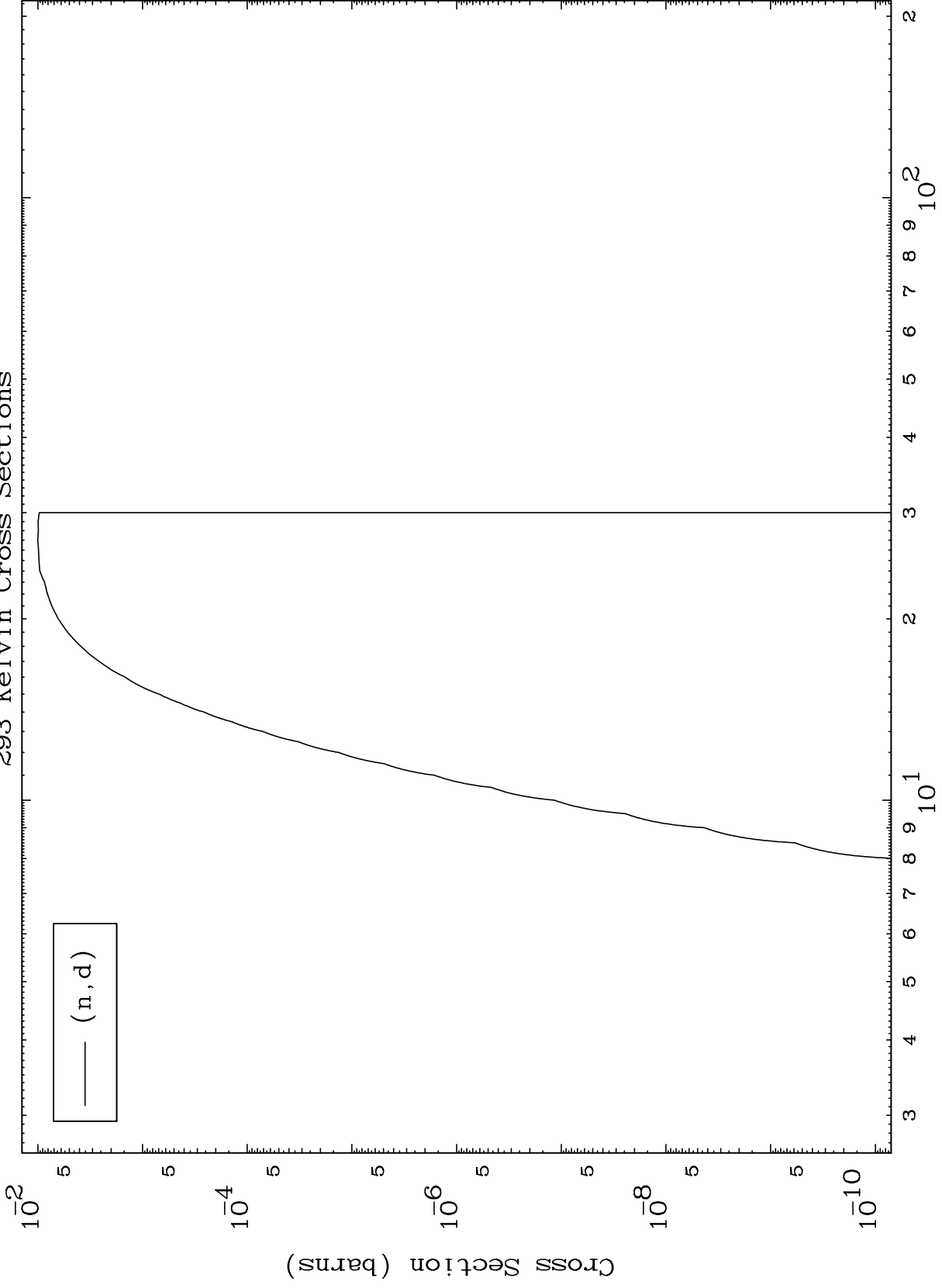


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(n,d) Levels
293 Kelvin Cross Sections

89-Ac-226



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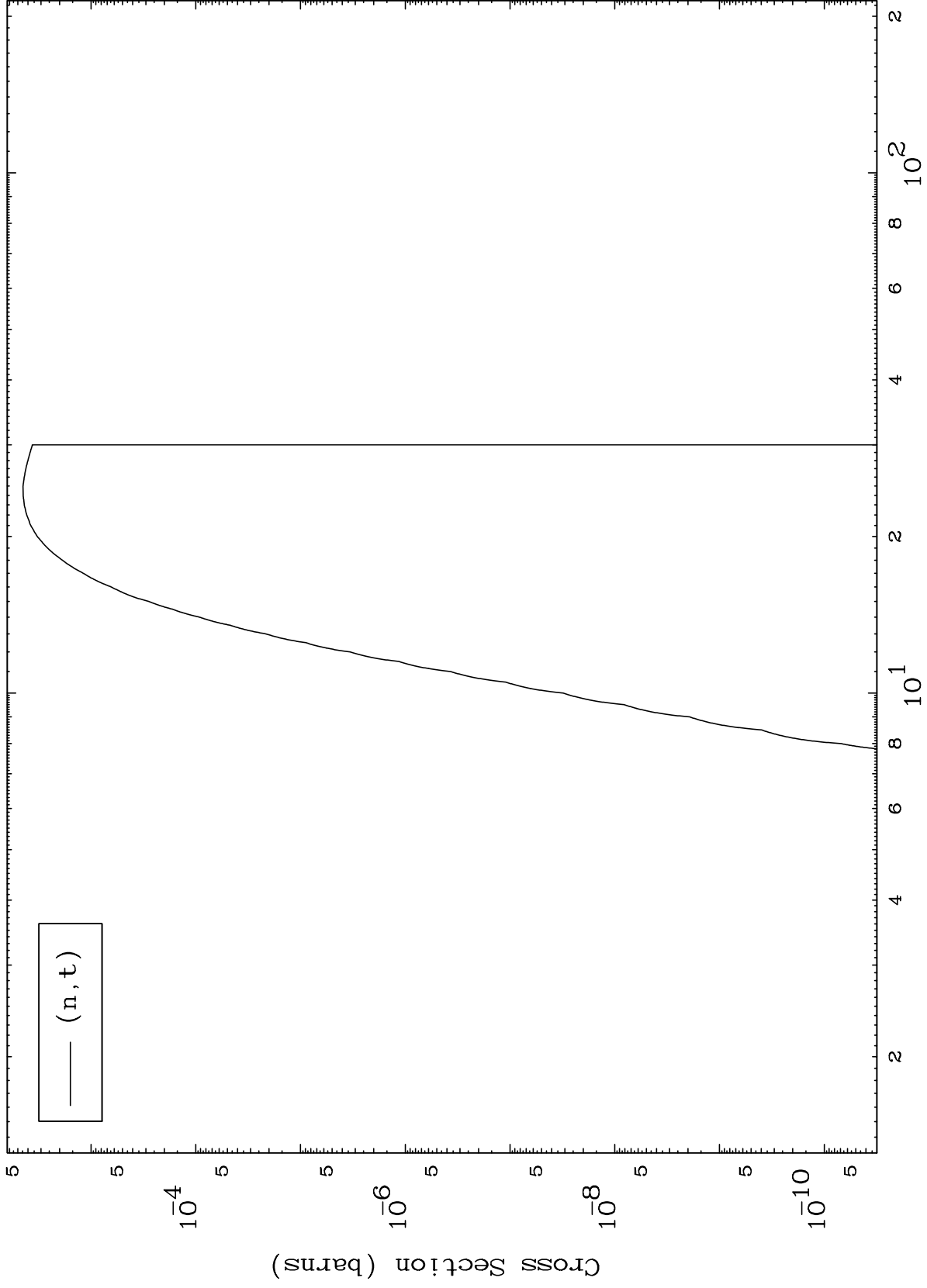
Incident Energy (MeV)

89-Ac-226

MAT 8928

(n,t) Levels
293 Kelvin Cross Sections

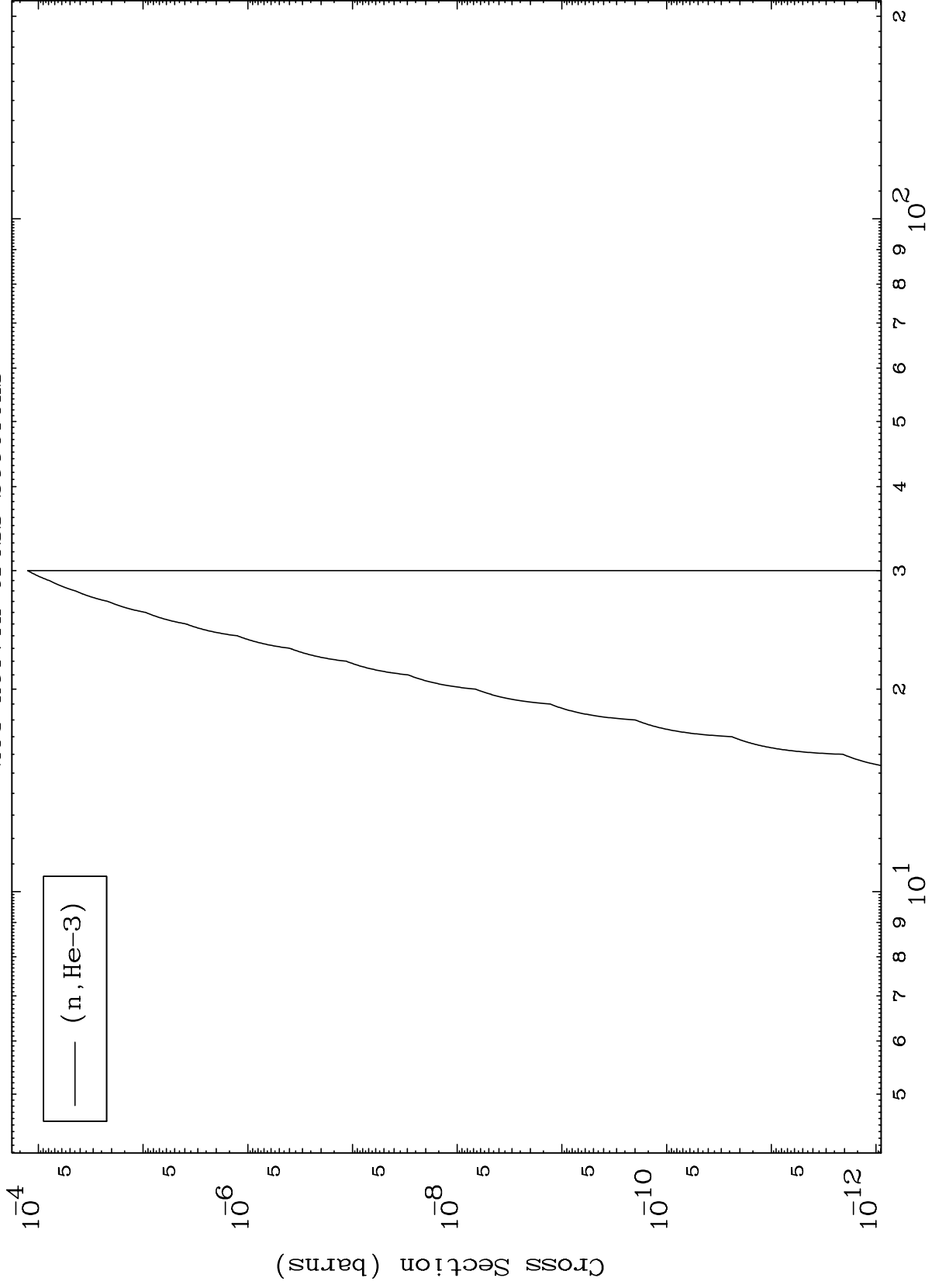
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(n,He3) Levels
293 Kelvin Cross Sections

89-Ac-226



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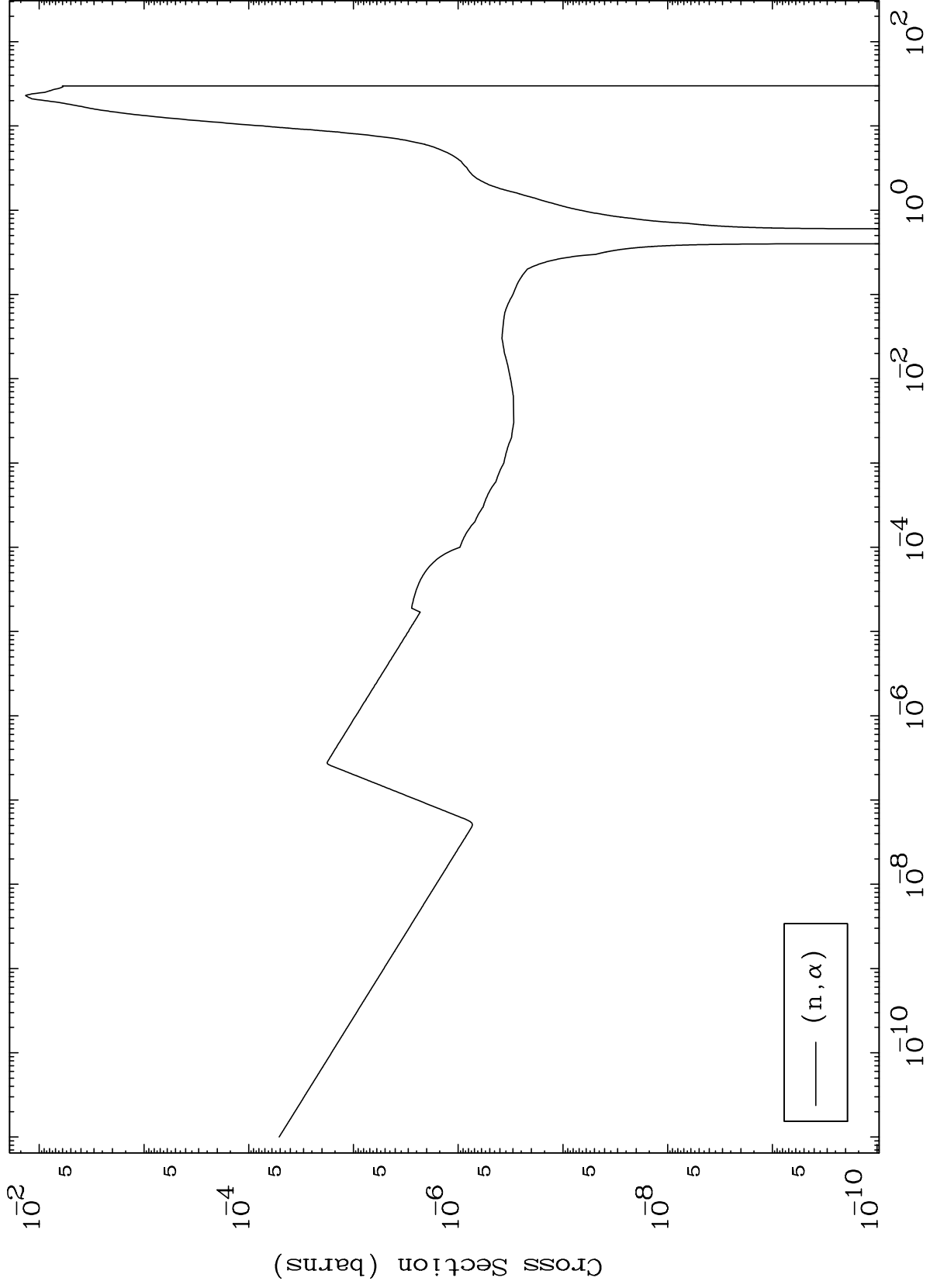
Incident Energy (MeV)

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MAT 8928

(n,α) Levels
293 Kelvin Cross Sections

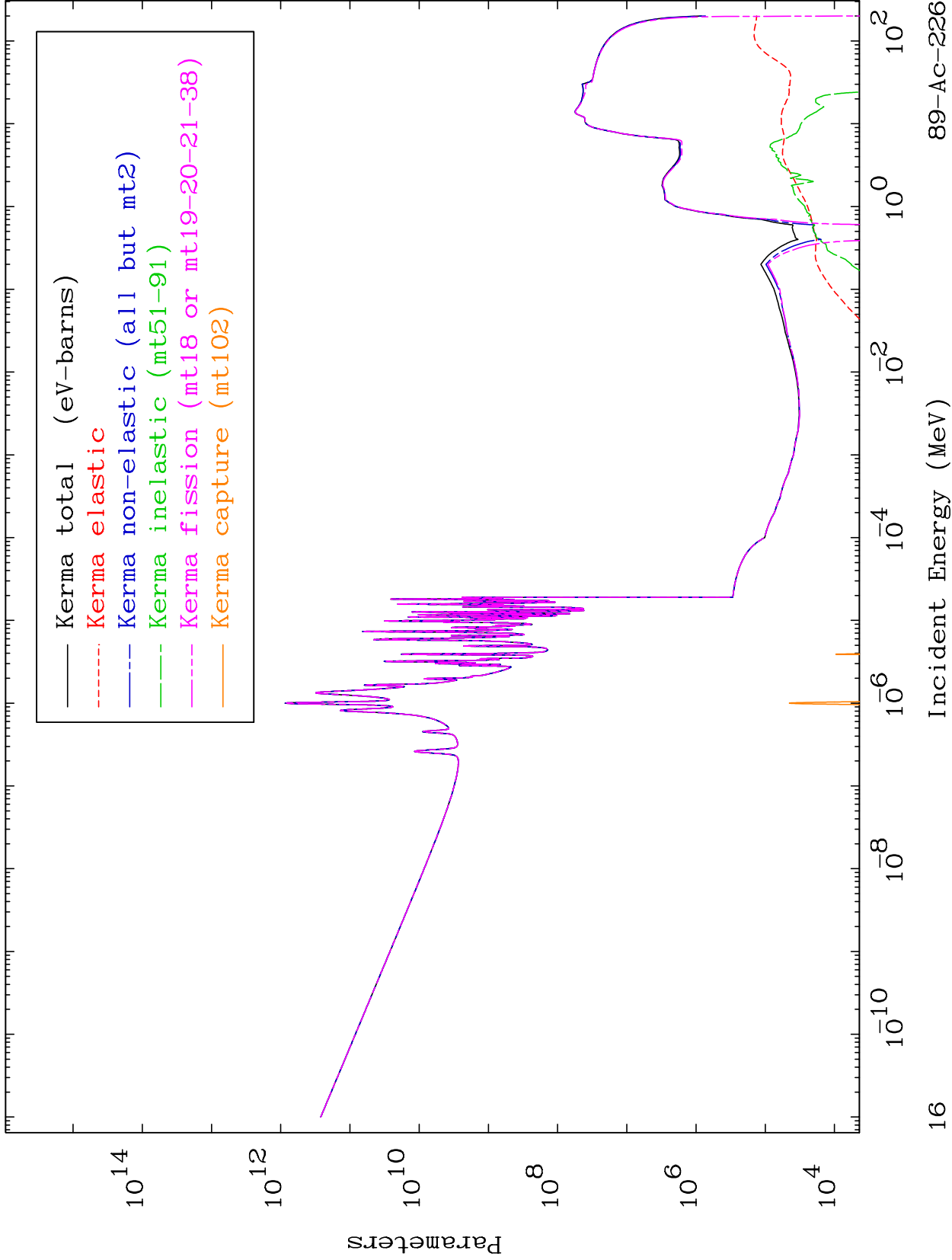
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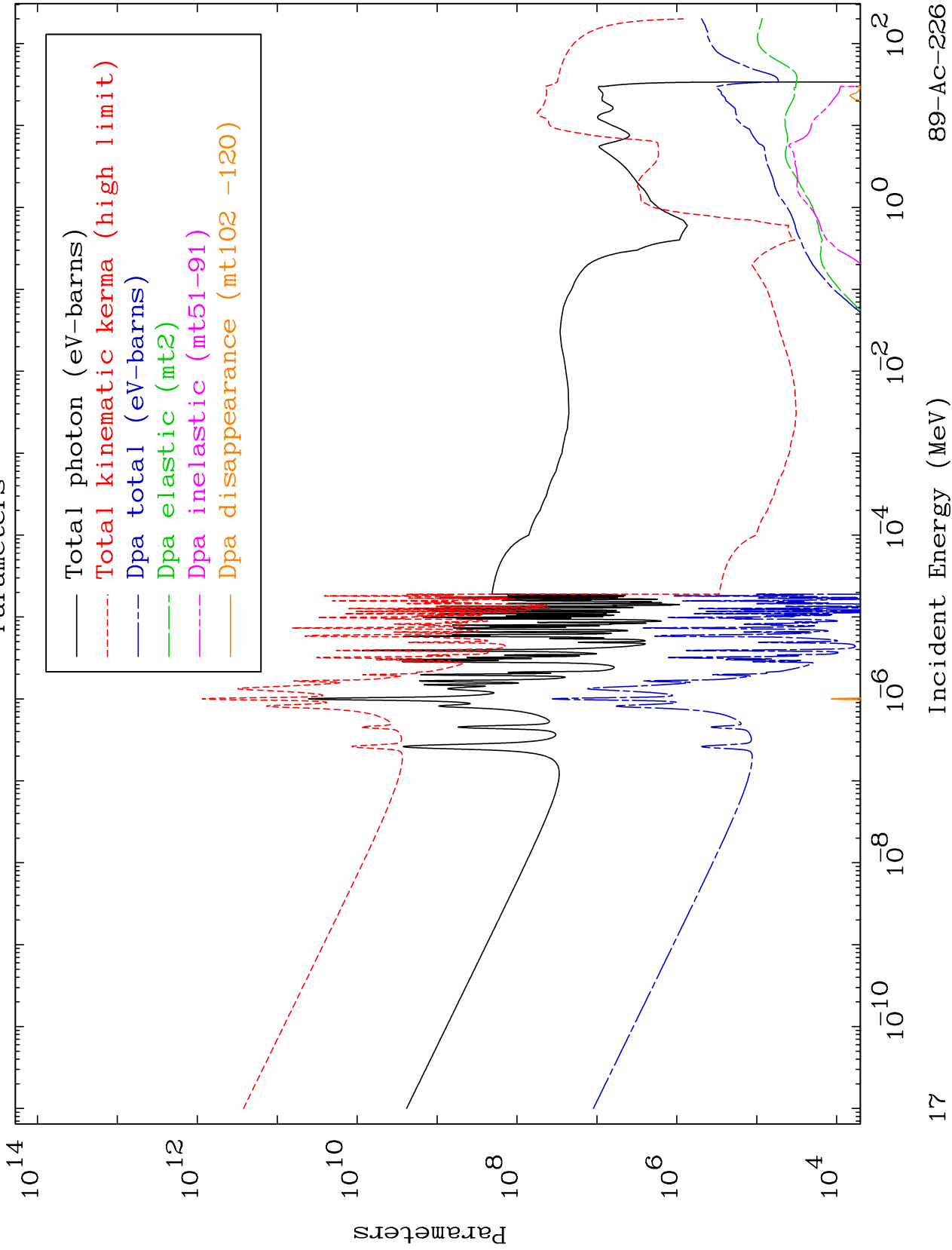


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Incident Energy (MeV)

89-Ac-226

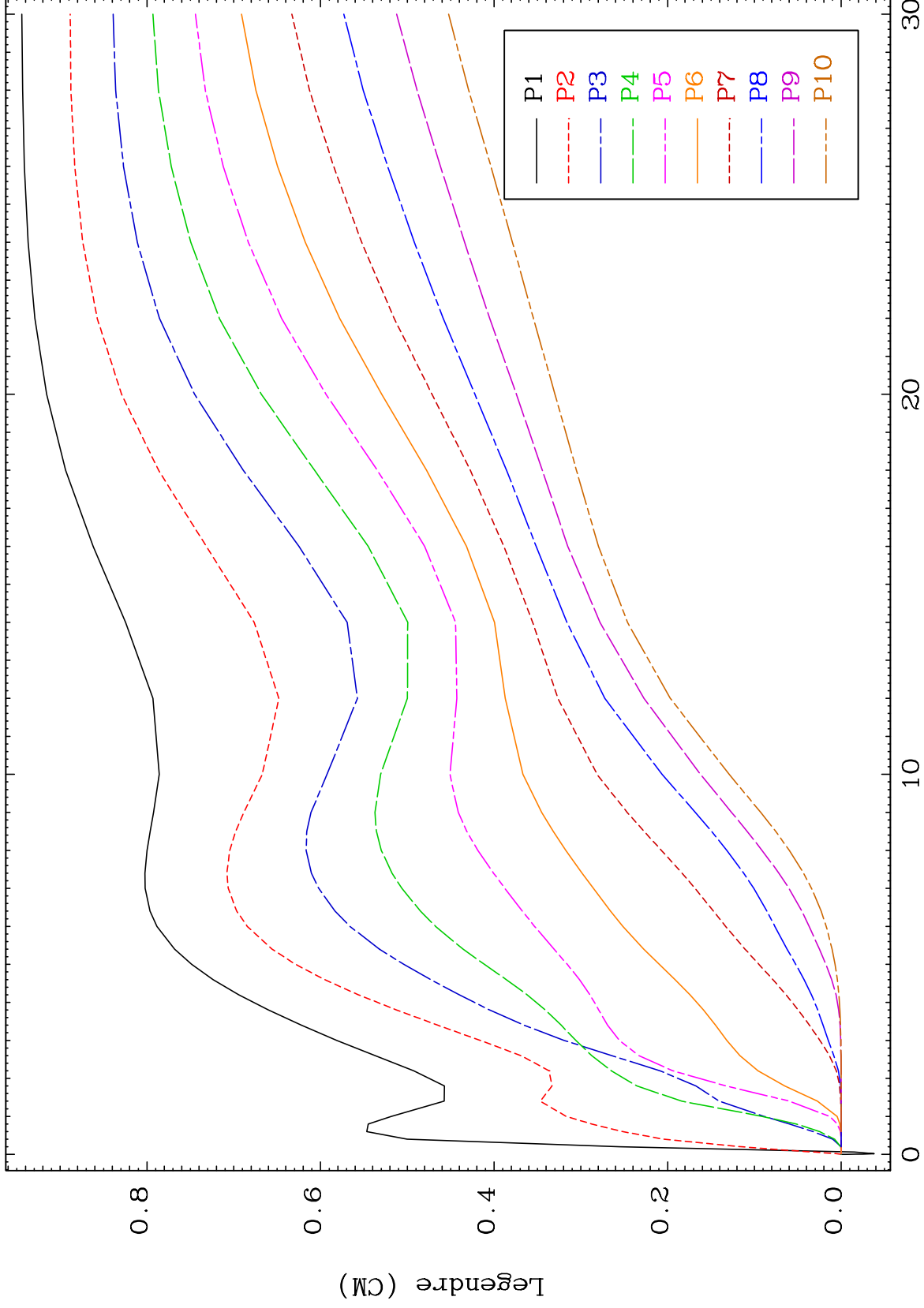




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89-Ac-226

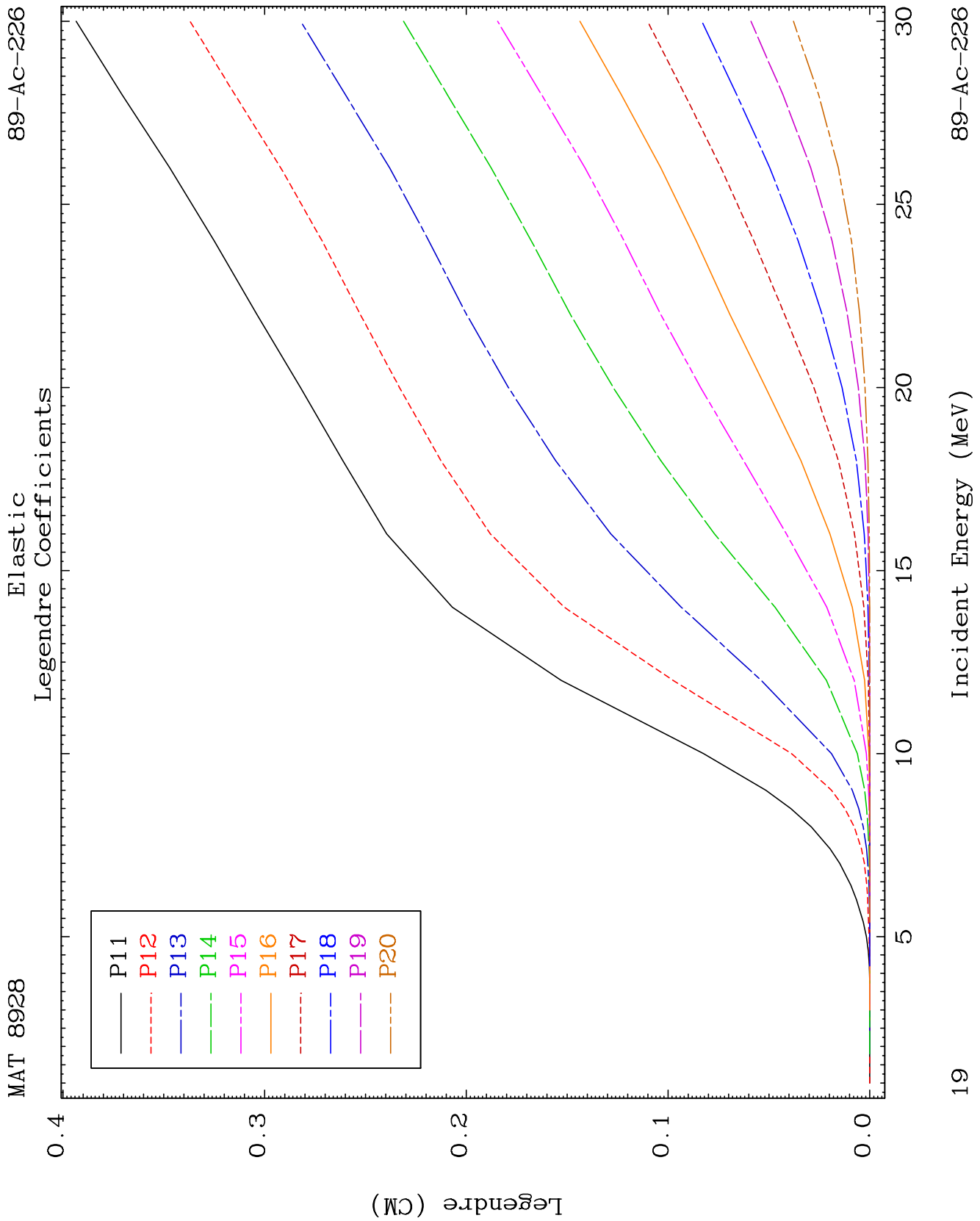
Elastic Legendre Coefficients



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Incident Energy (MeV)

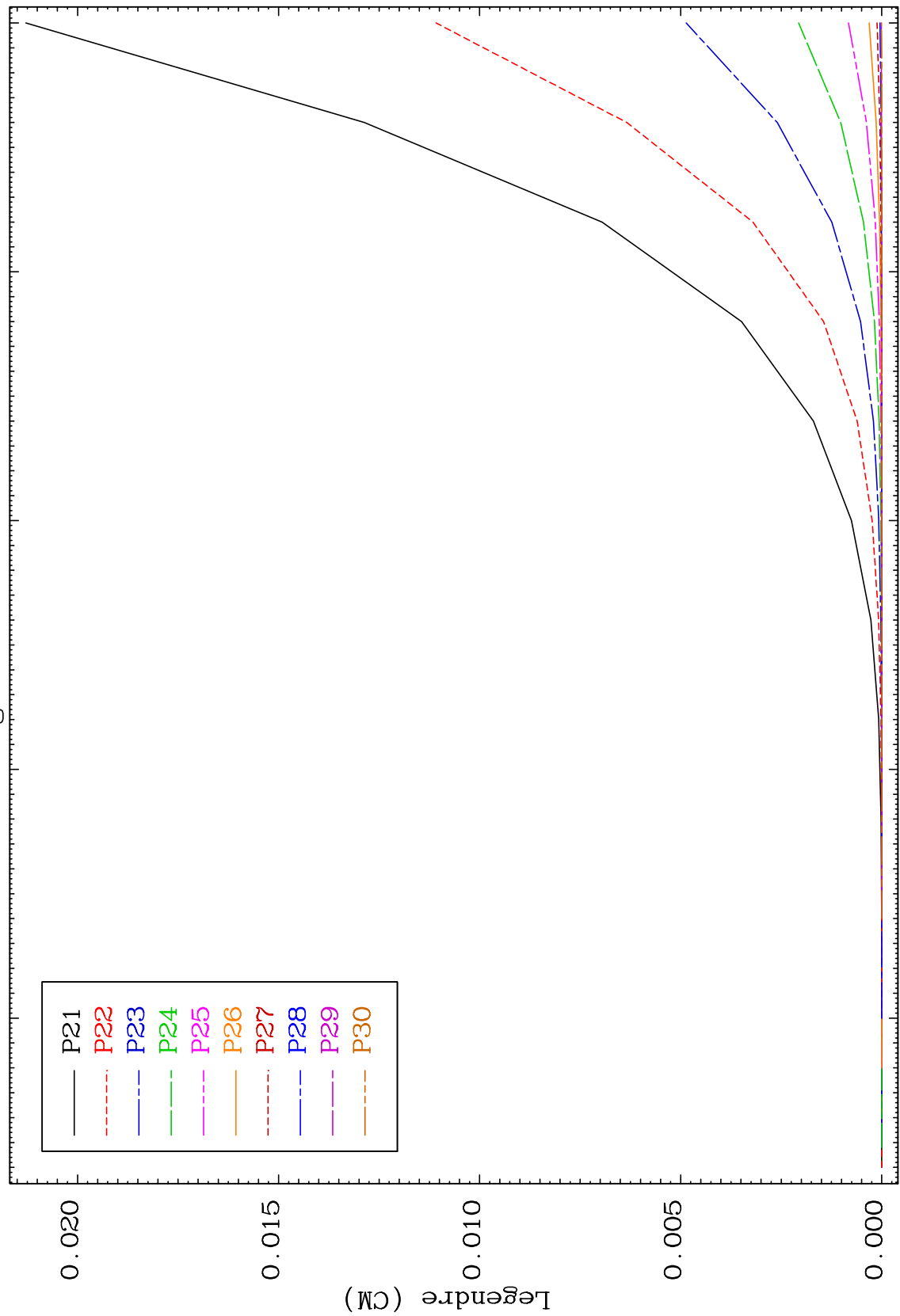
89-Ac-226



MAT 8928

Elastic Legendre Coefficients

89-Ac-226



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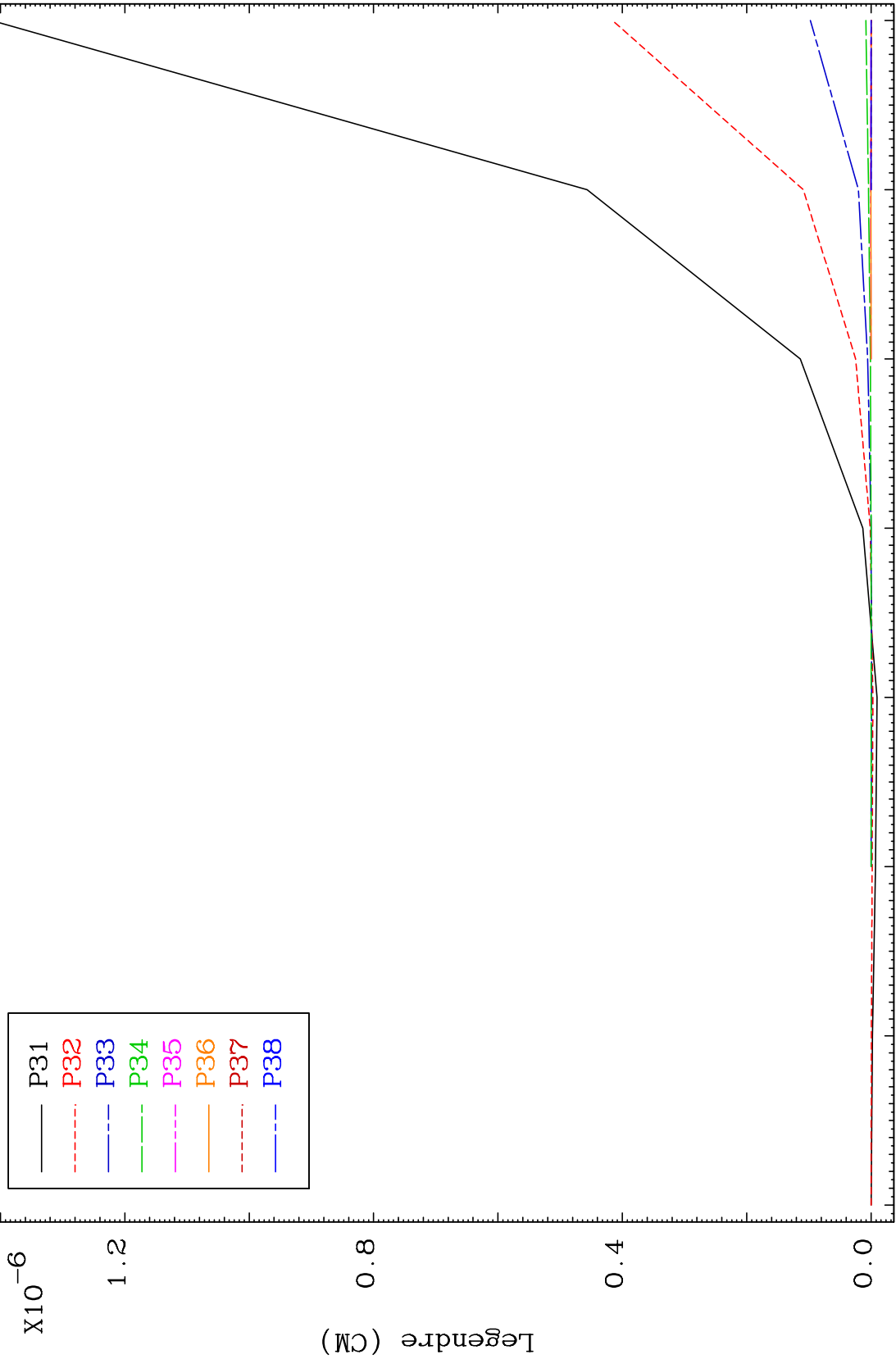
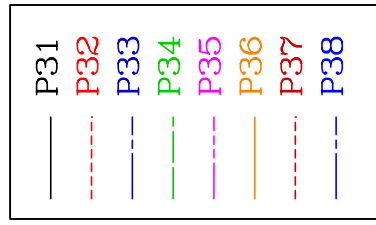
Incident Energy (MeV)

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Elastic
Legendre Coefficients

89-Ac-226

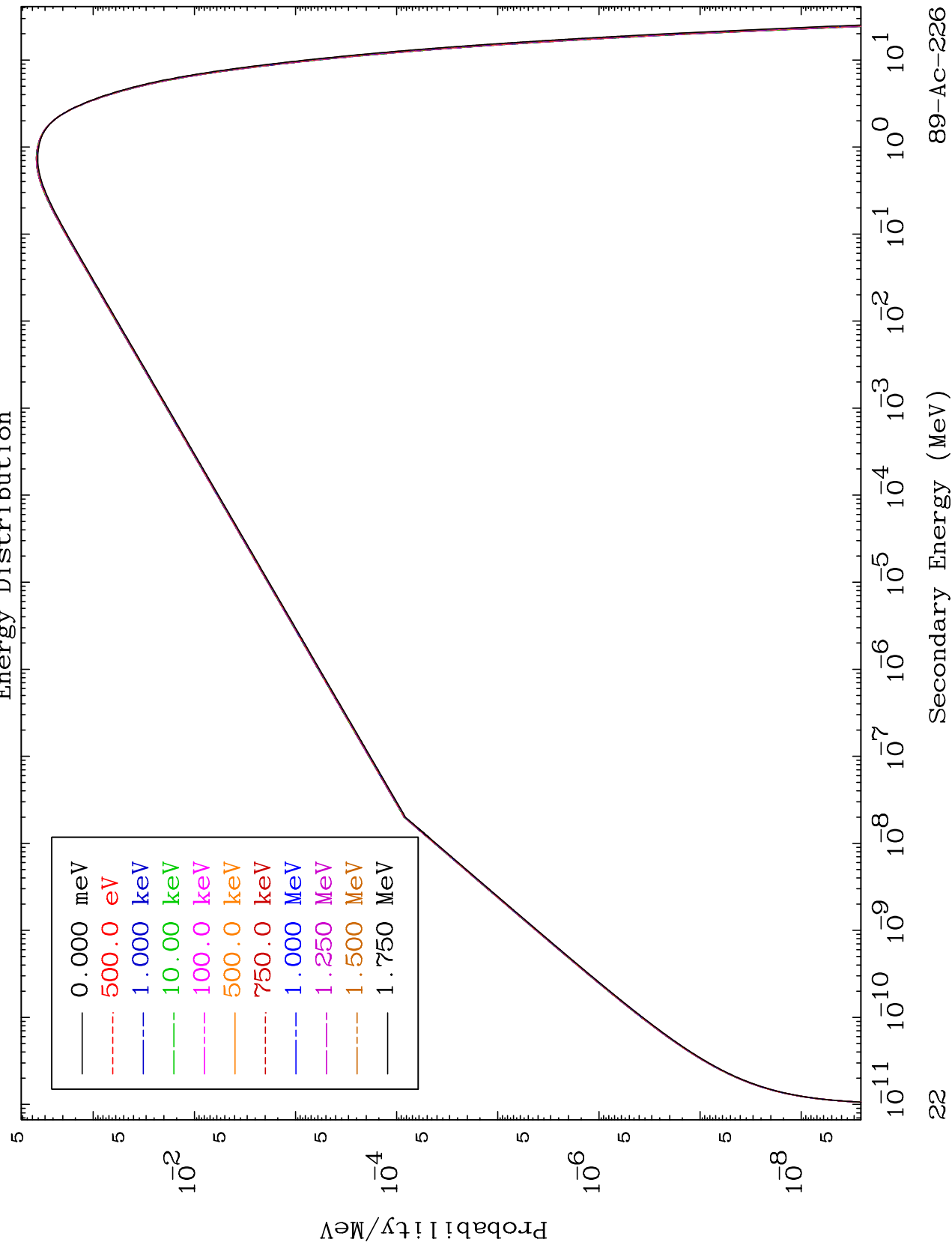


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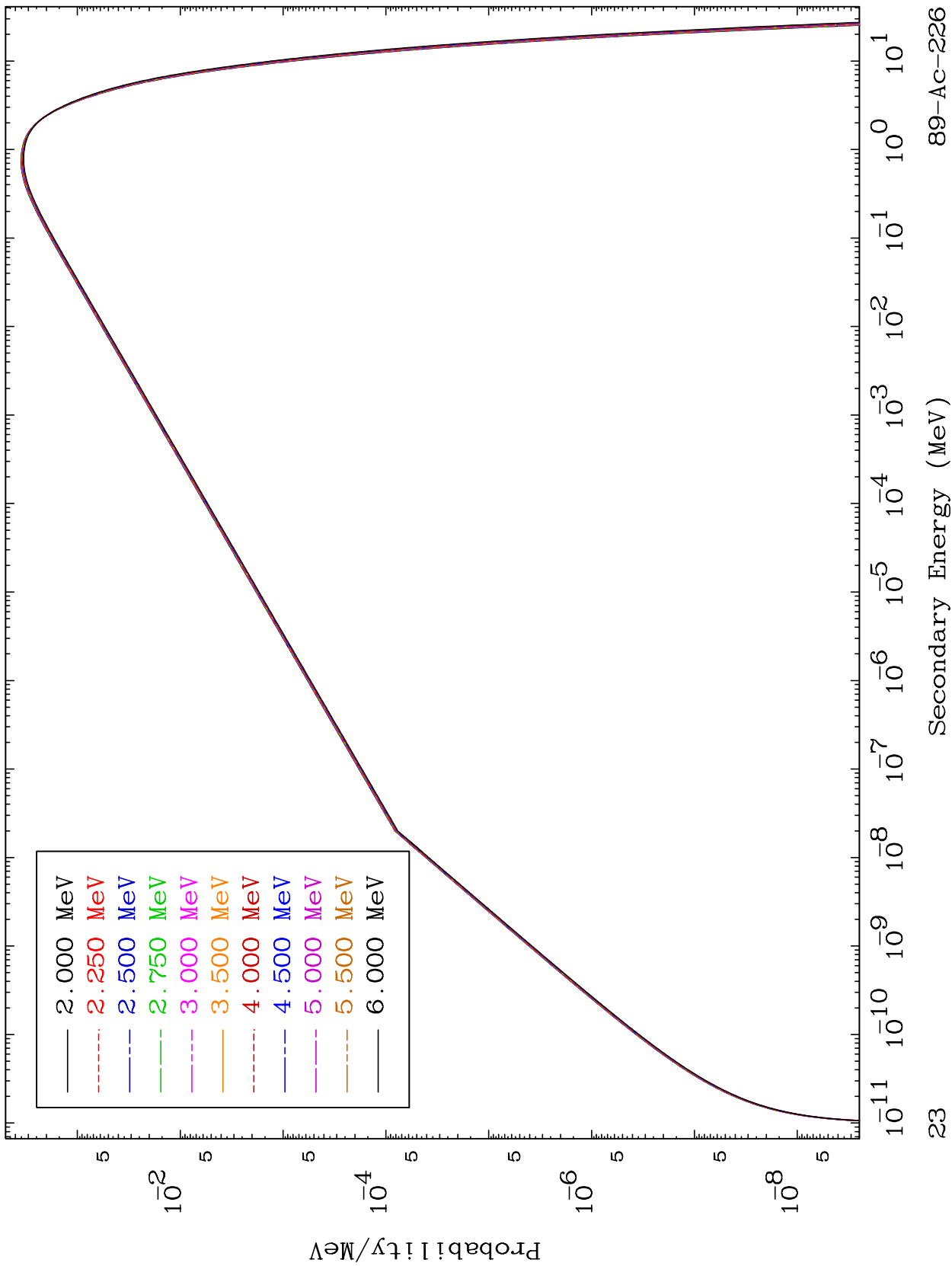
Incident Energy (MeV)

89-Ac-226

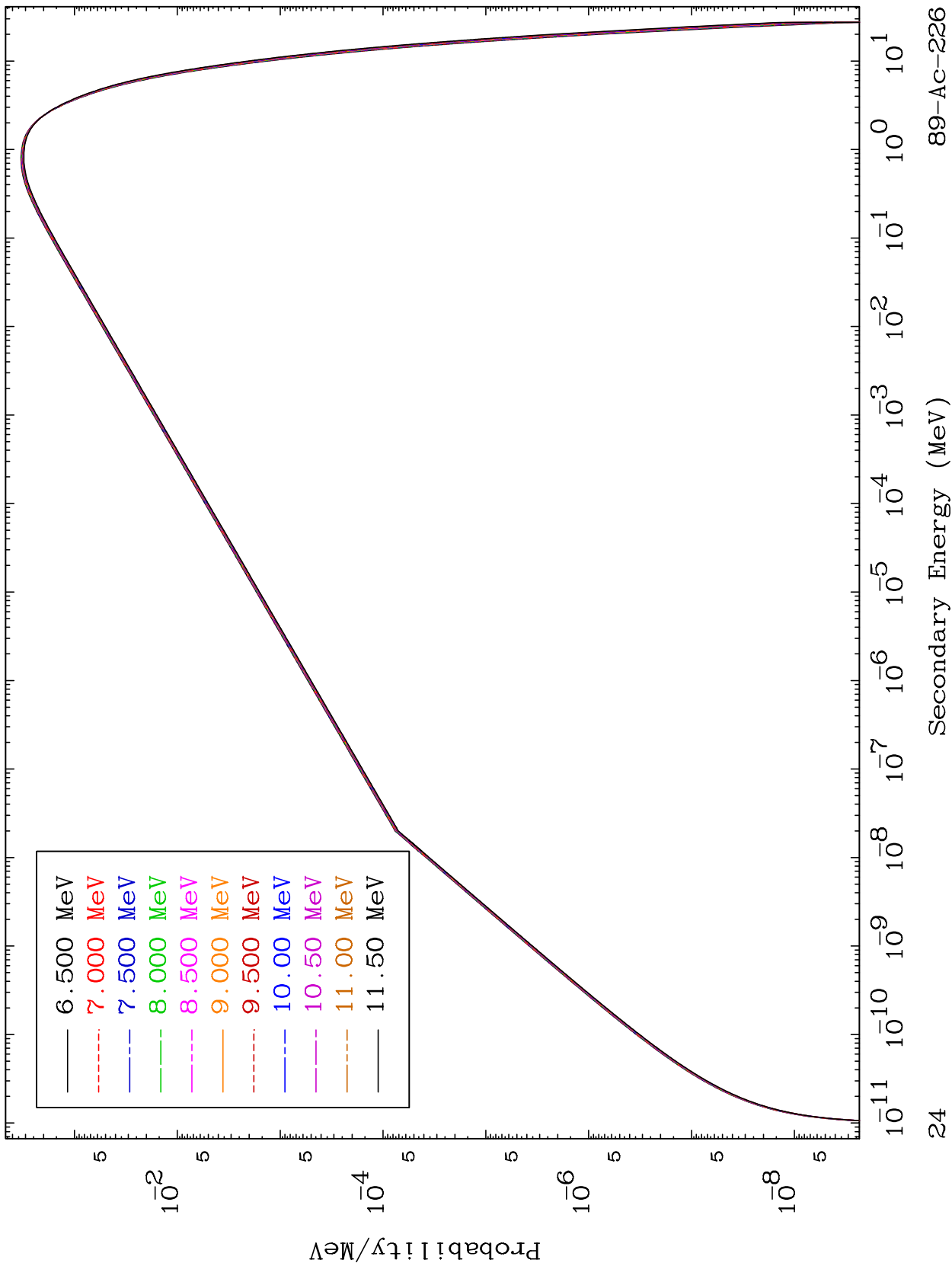
Fission Energy Distribution



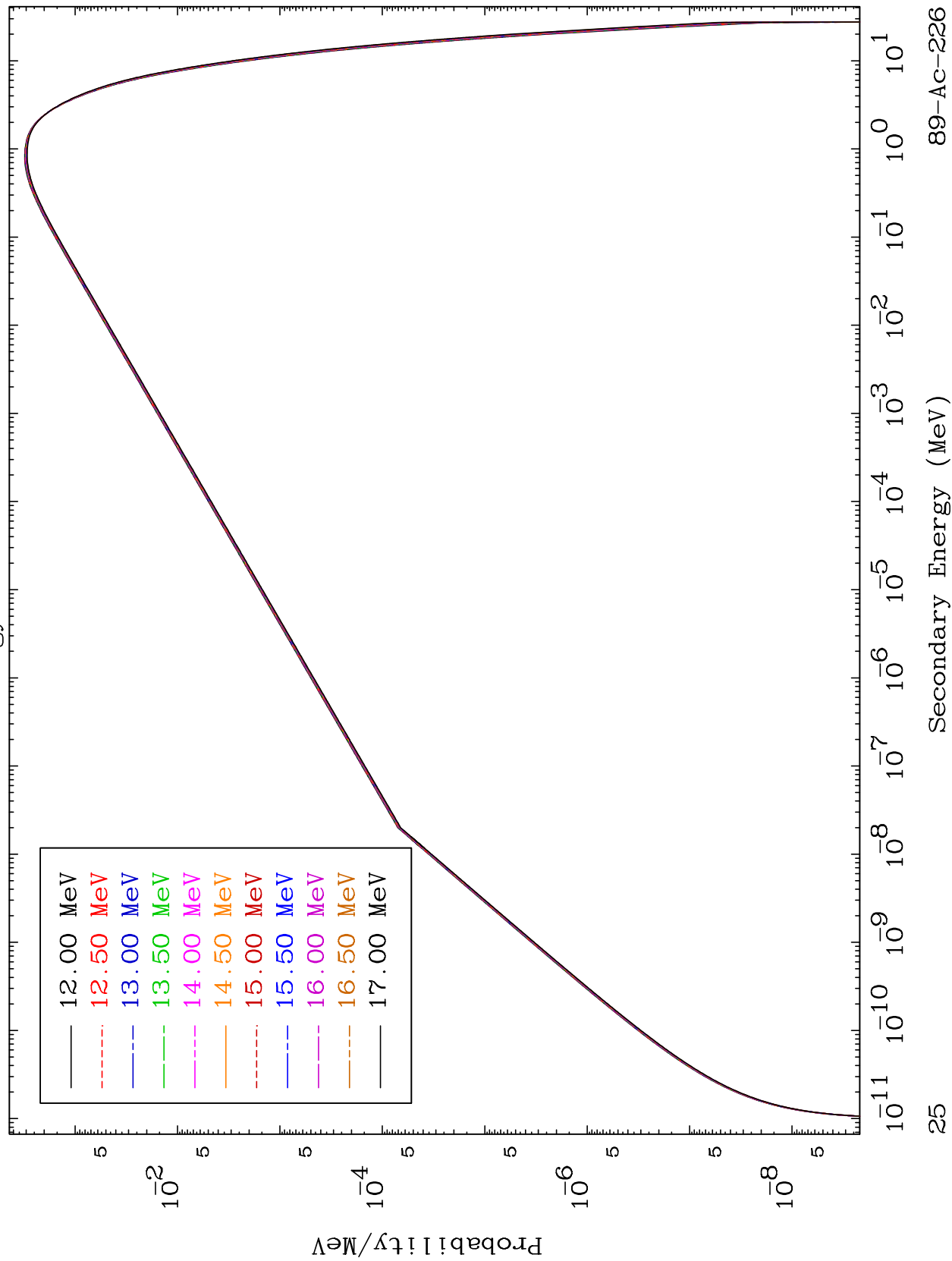
Fission Energy Distribution



Fission Energy Distribution



Fission Energy Distribution



Fission Energy Distribution

