

Program EVALPLOT
(Version 2018-1)

by

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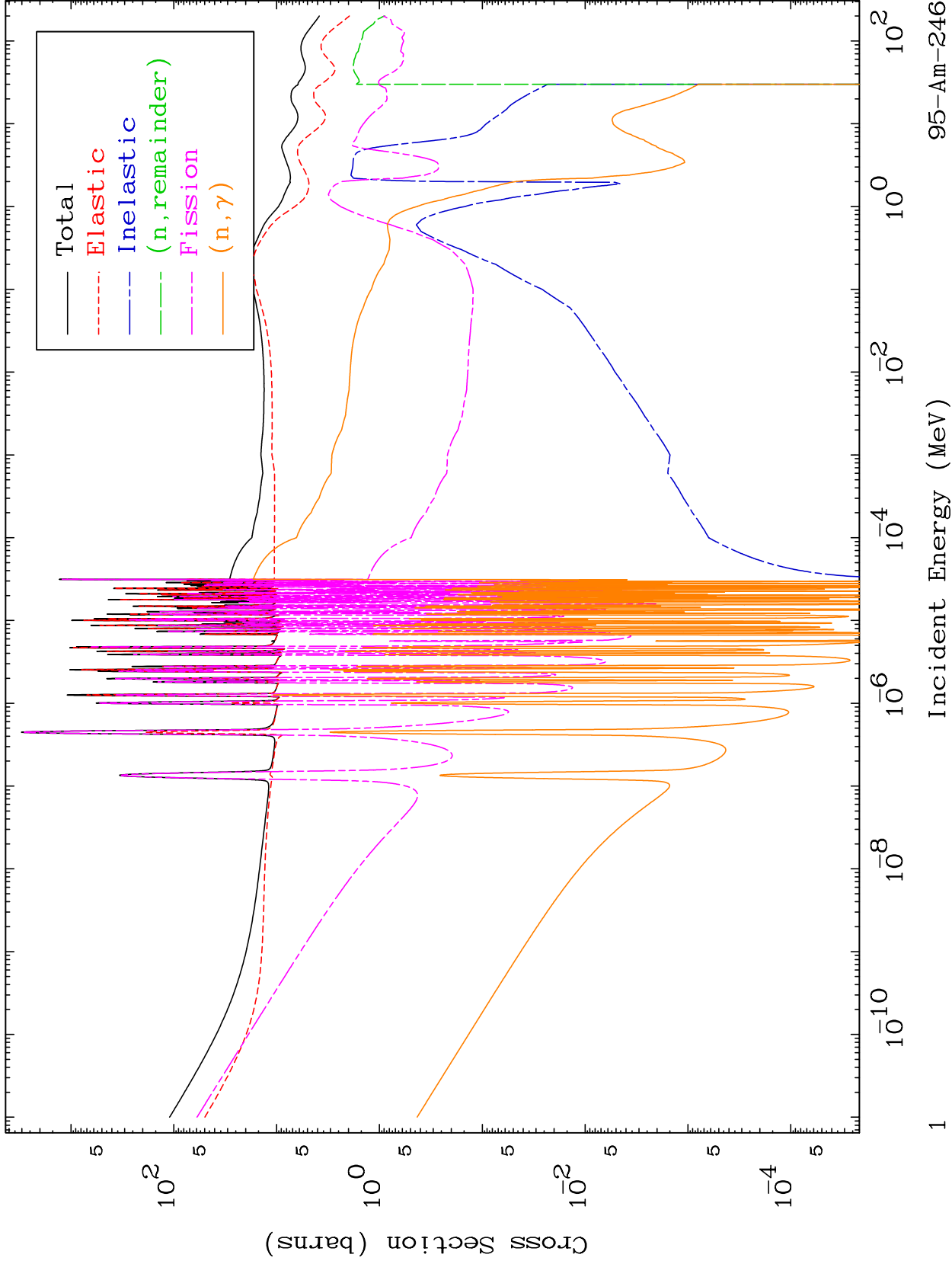
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9559

Major
293 Kelvin Cross Sections

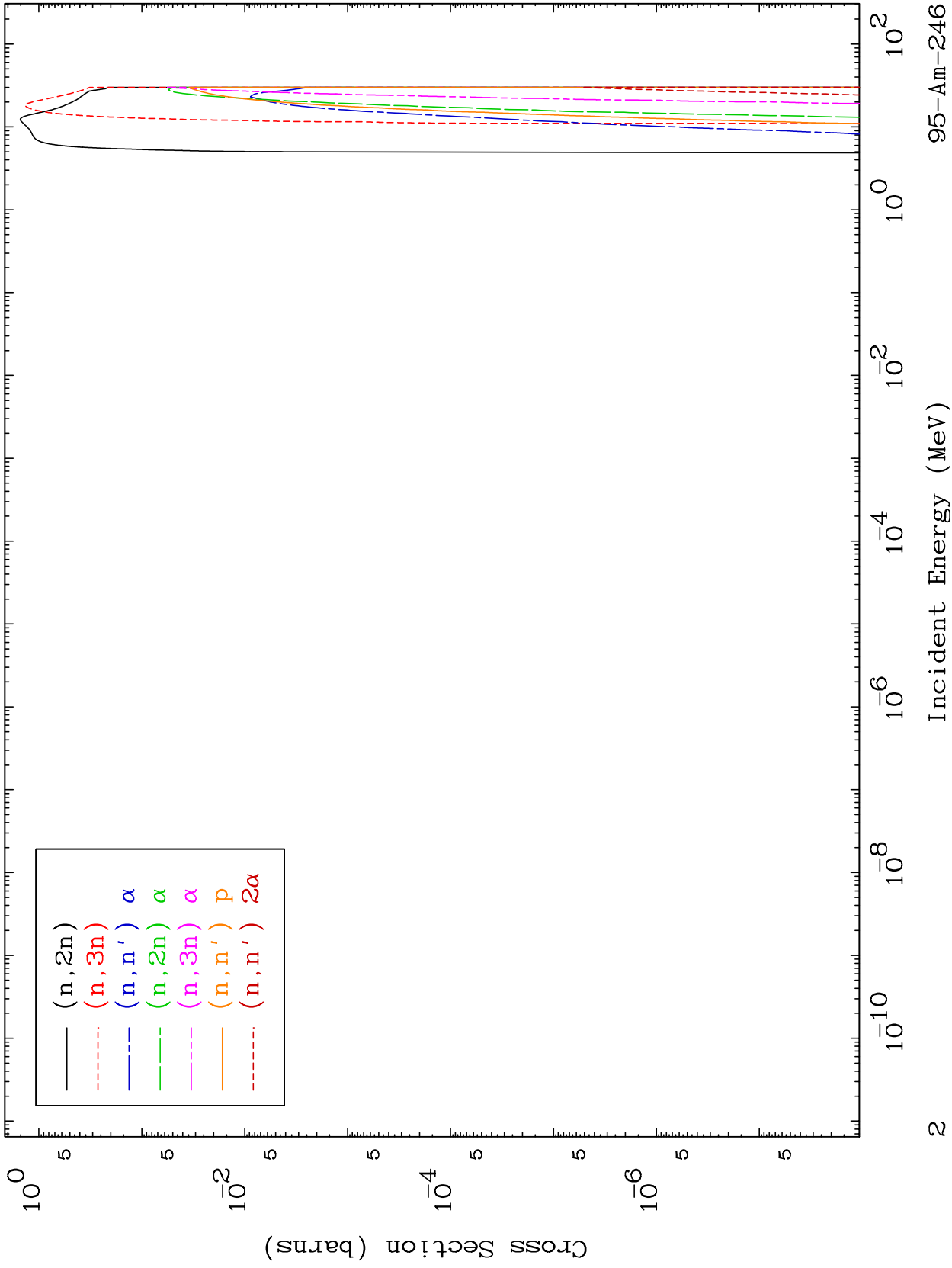
95-Am-246



MAT 9559

Neutron Production
293 Kelvin Cross Sections

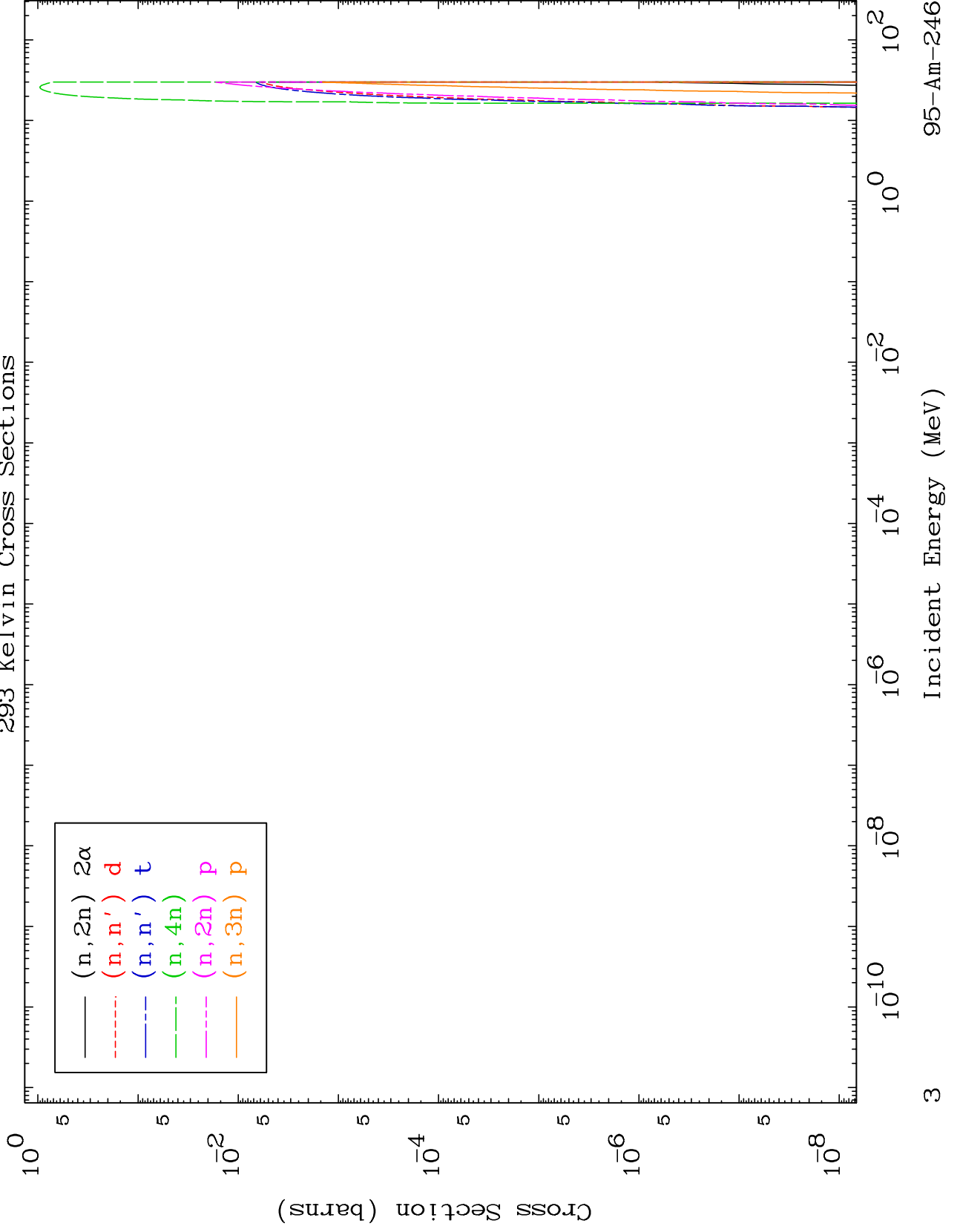
95-Am-246



MAT 9559

Neutron Production
293 Kelvin Cross Sections

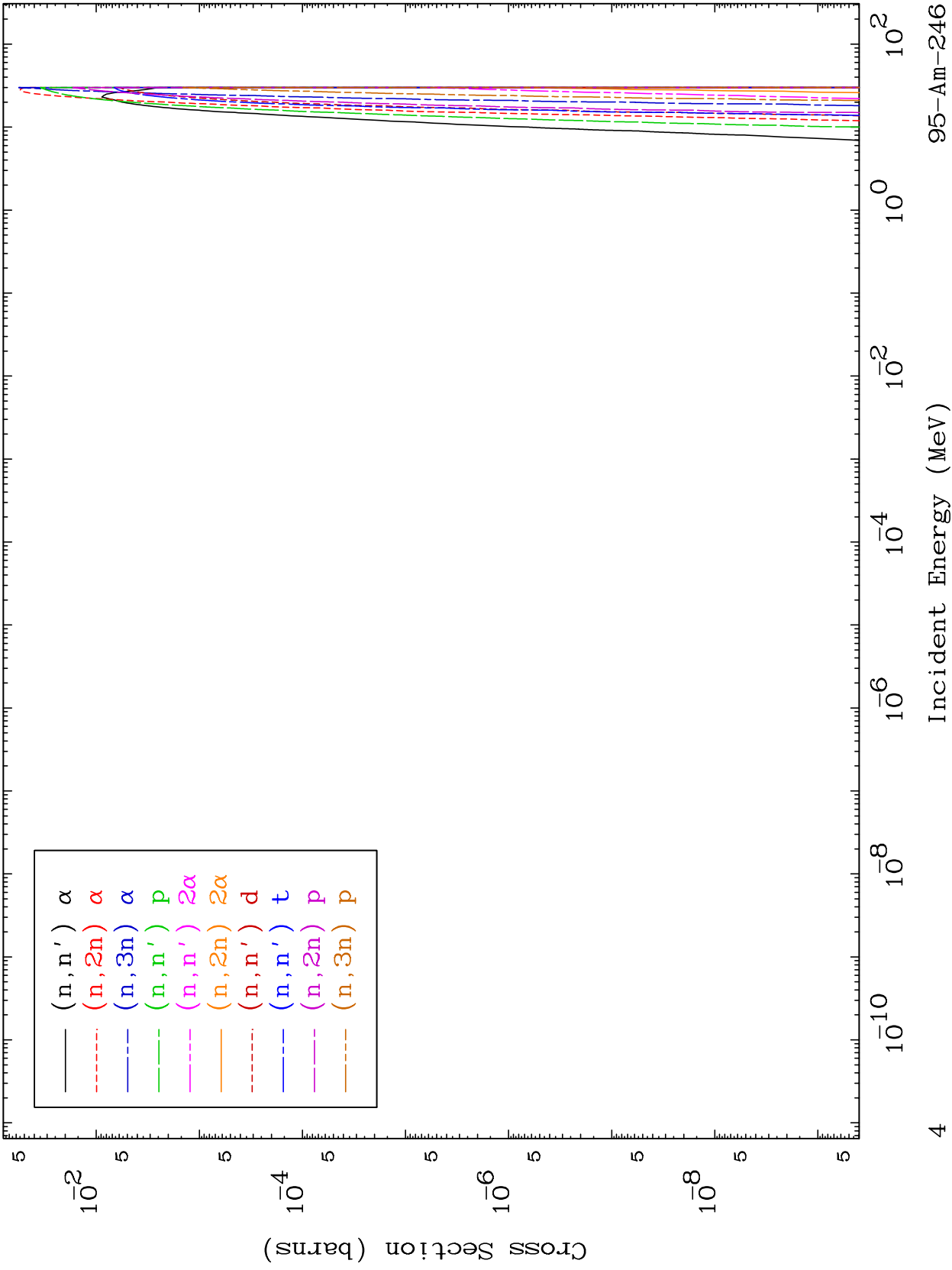
95-Am-246



MAT 9559

Charged Particle
293 Kelvin Cross Sections

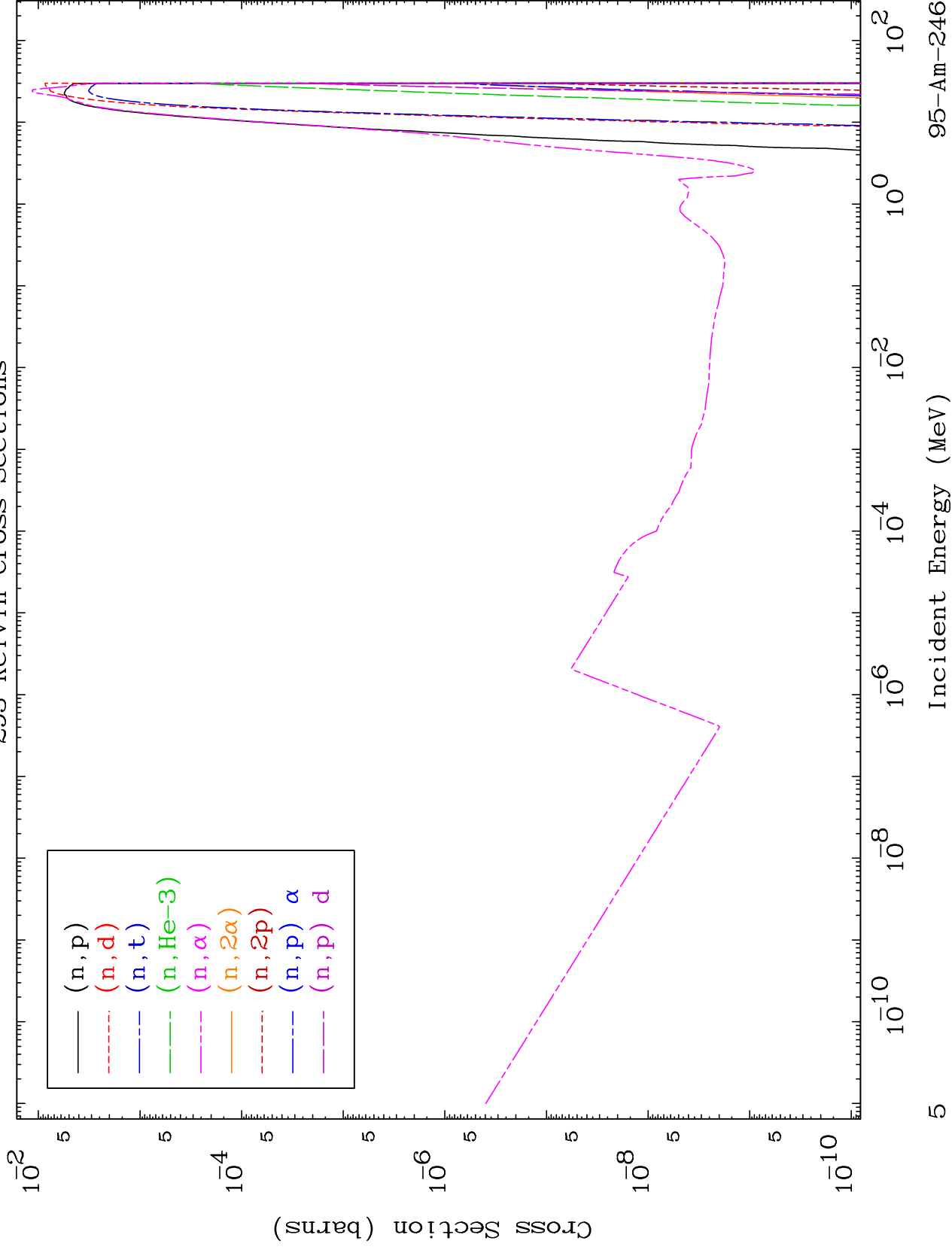
95-Am-246



MAT 9559

Charged Particle
293 Kelvin Cross Sections

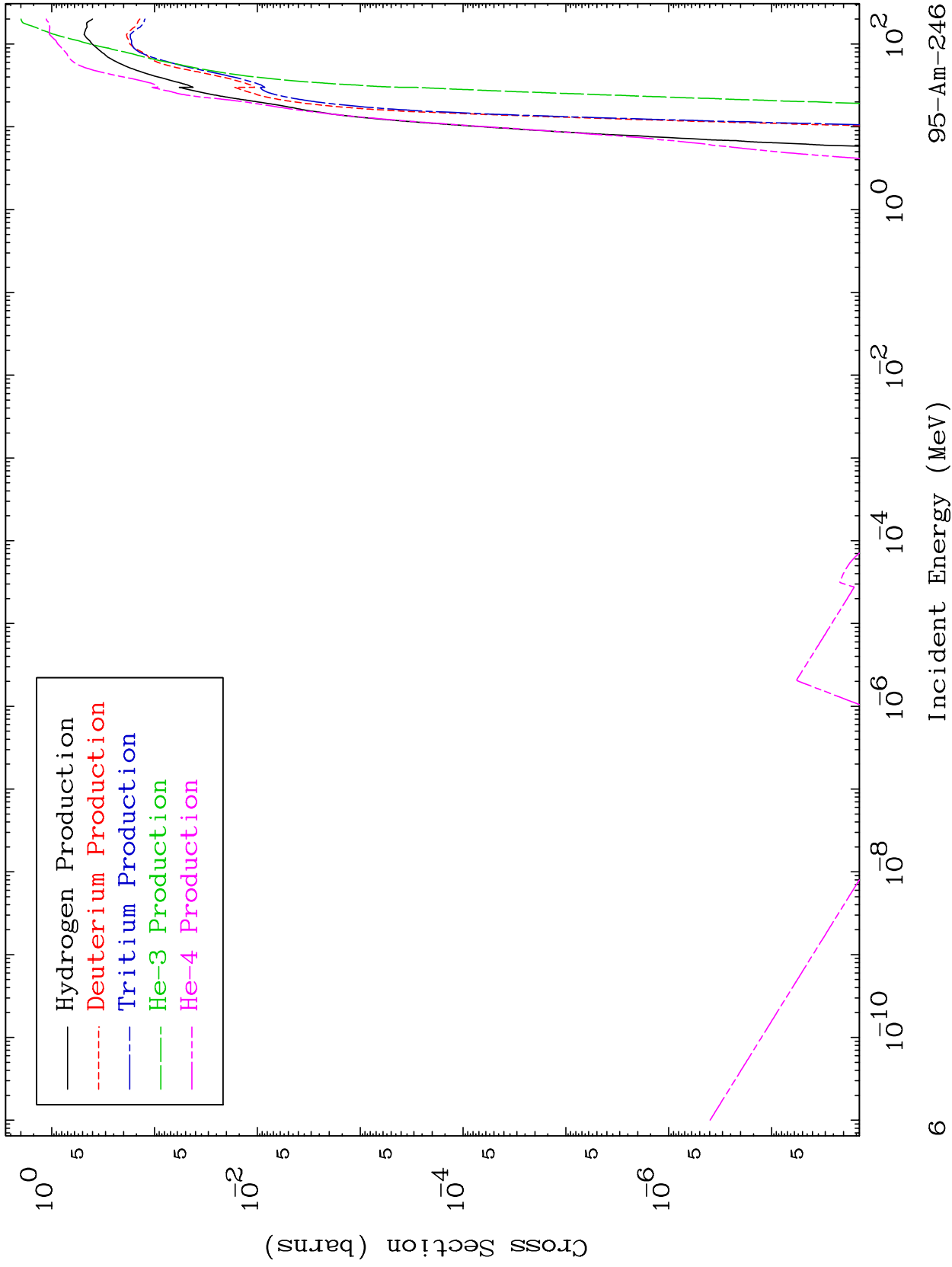
95-Am-246



MAT 9559

Particle Production
293 Kelvin Cross Sections

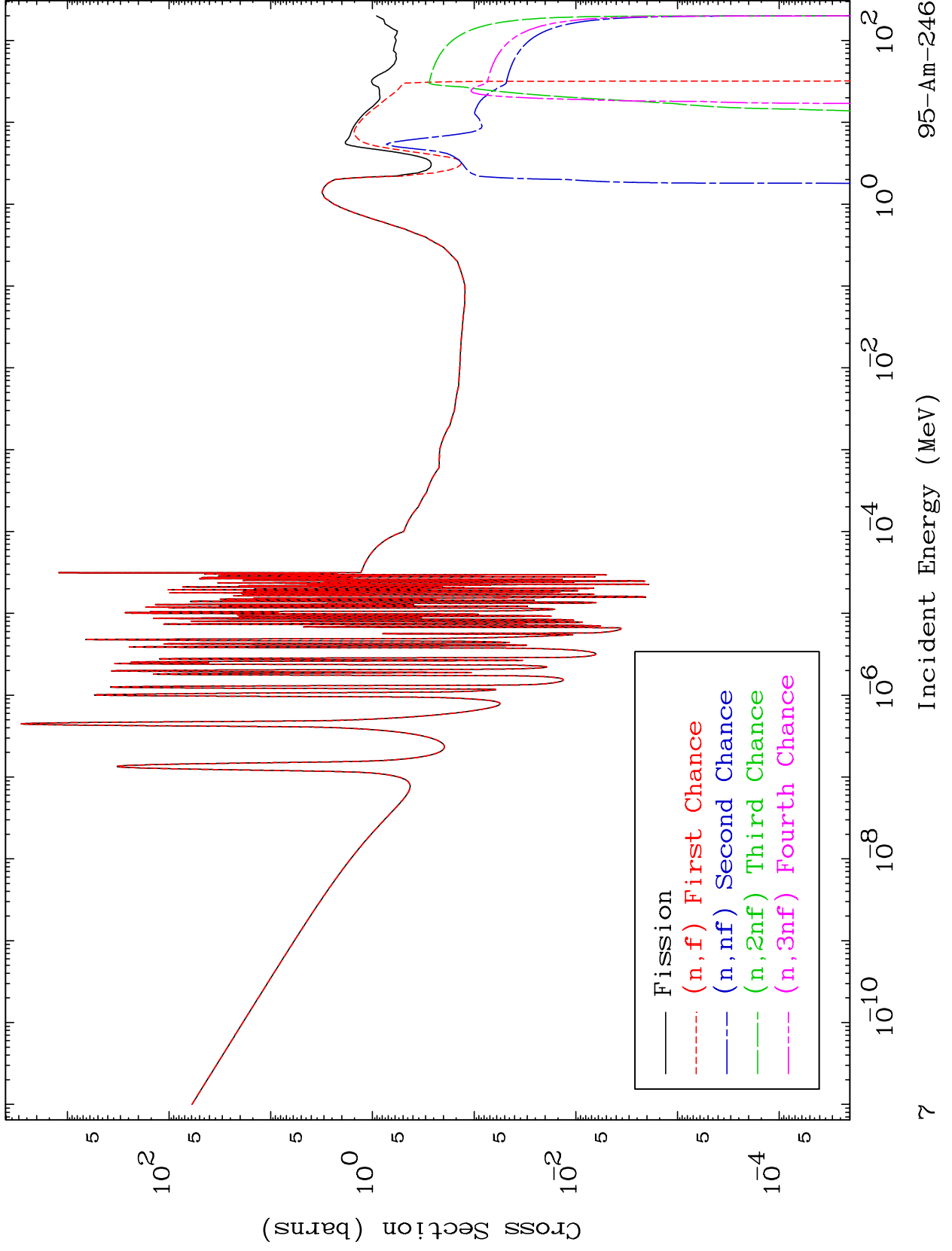
95-Am-246



MAT 9559

Fission
293 Kelvin Cross Sections

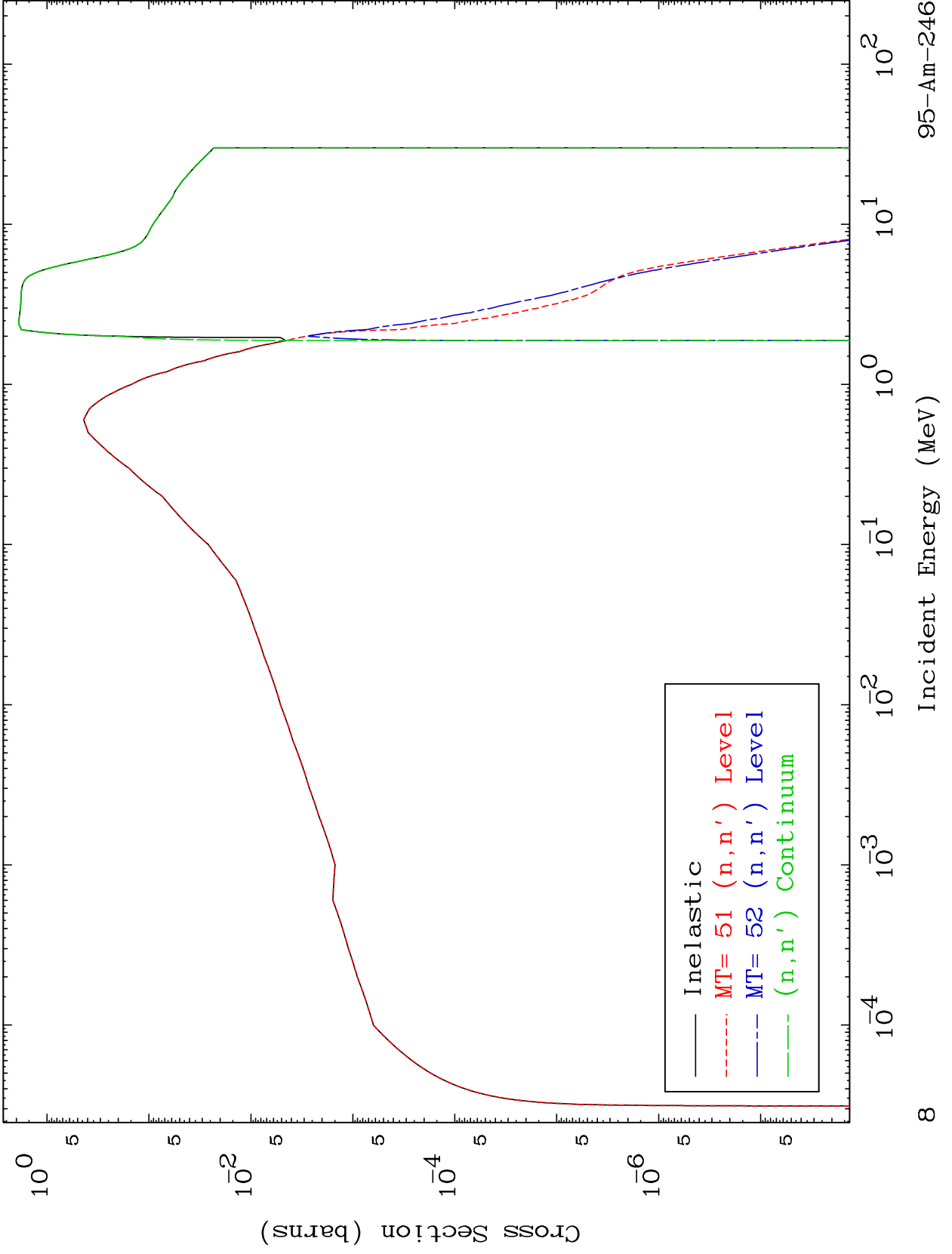
95-Am-246



MAT 9559

(n,n') Level
293 Kelvin Cross Sections

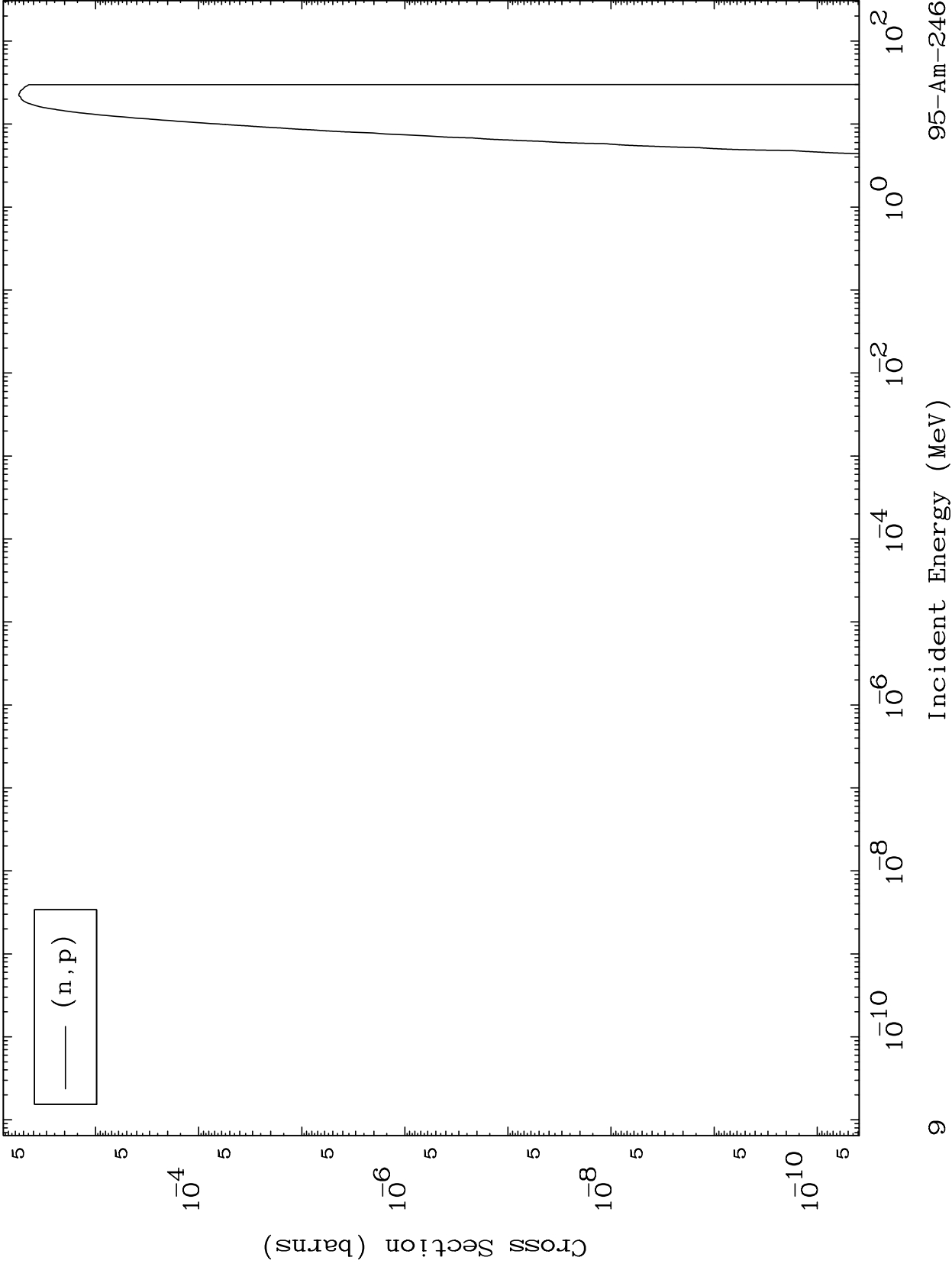
95-Am-246



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(n,p) Levels
293 Kelvin Cross Sections

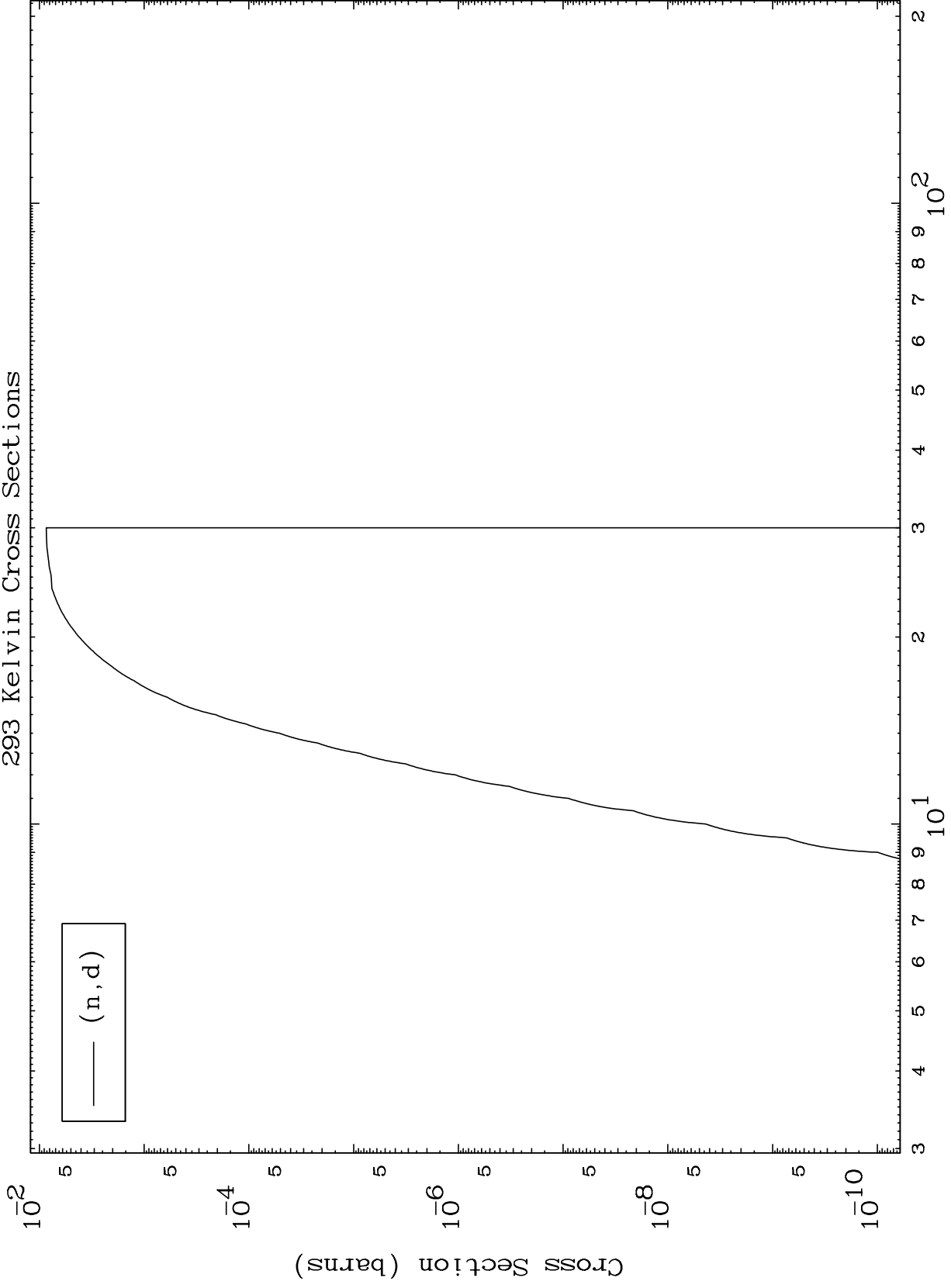
95-Am-246



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(n,d) Levels
293 Kelvin Cross Sections

95-Am-246



10

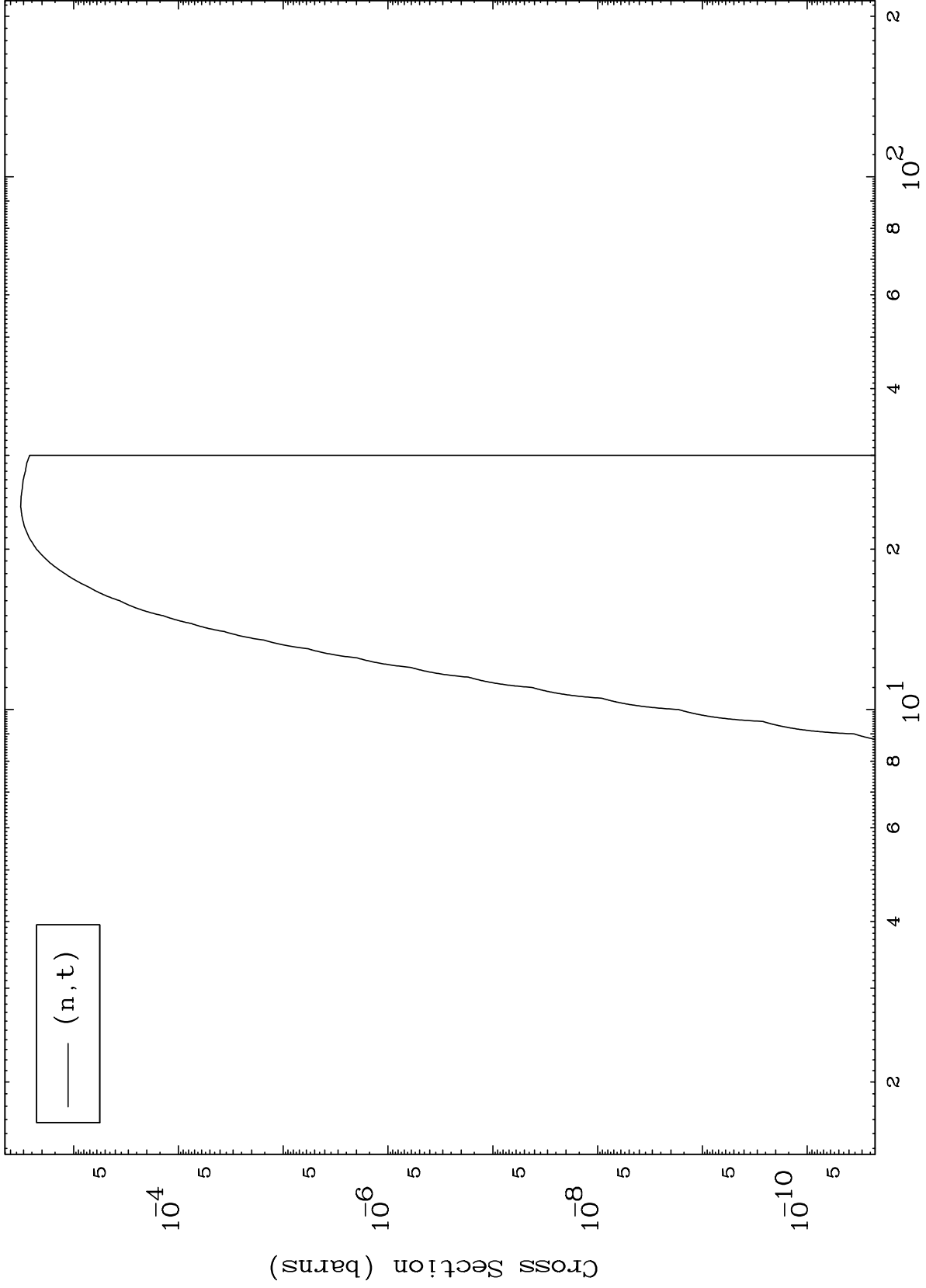
Incident Energy (MeV)

95-Am-246

MAT 9559

(n,t) Levels
293 Kelvin Cross Sections

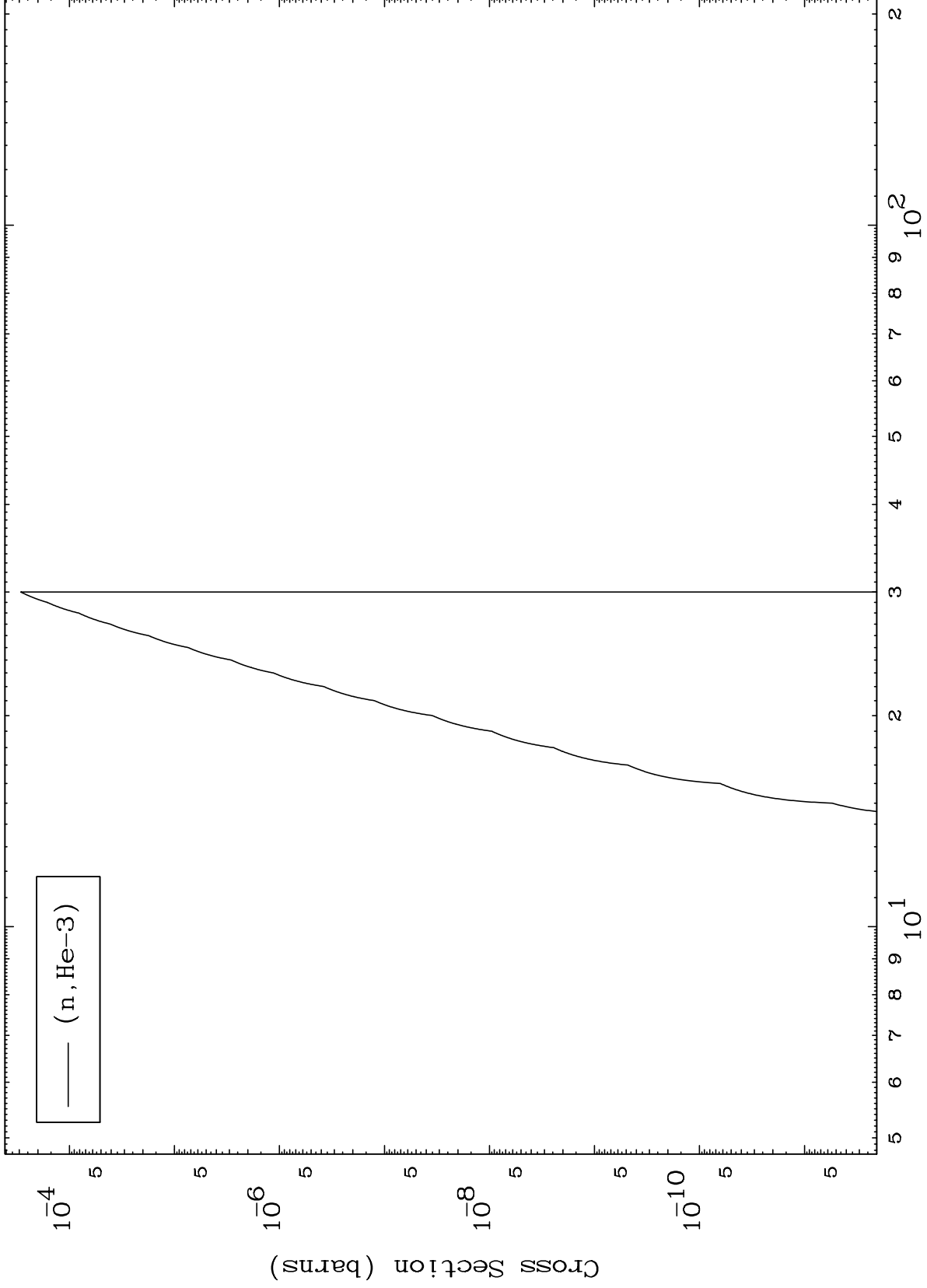
95-Am-246



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(n,He3) Levels
293 Kelvin Cross Sections

95-Am-246



12

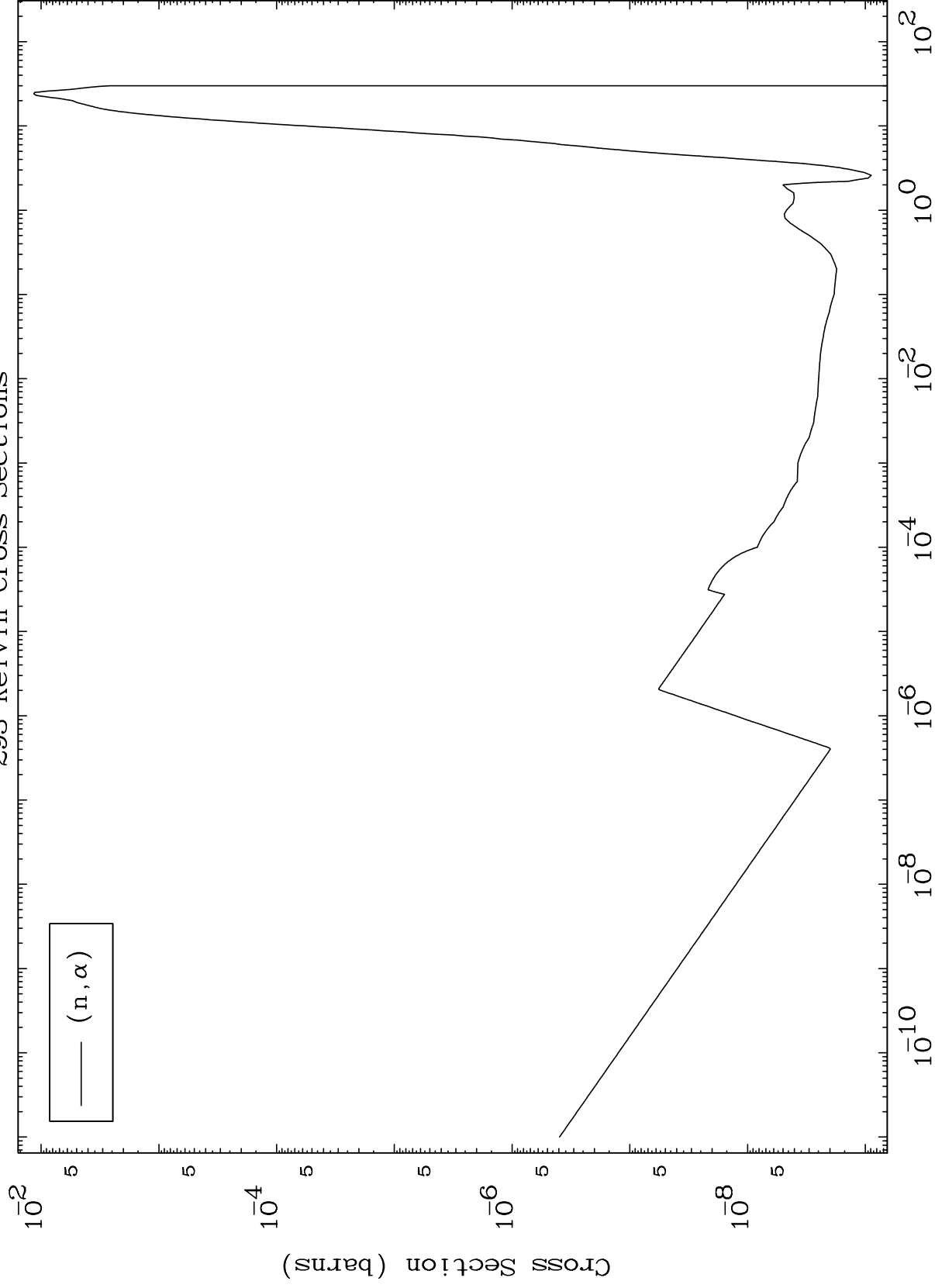
Incident Energy (MeV)

95-Am-246

MAT 9559

(n, α) Levels
293 Kelvin Cross Sections

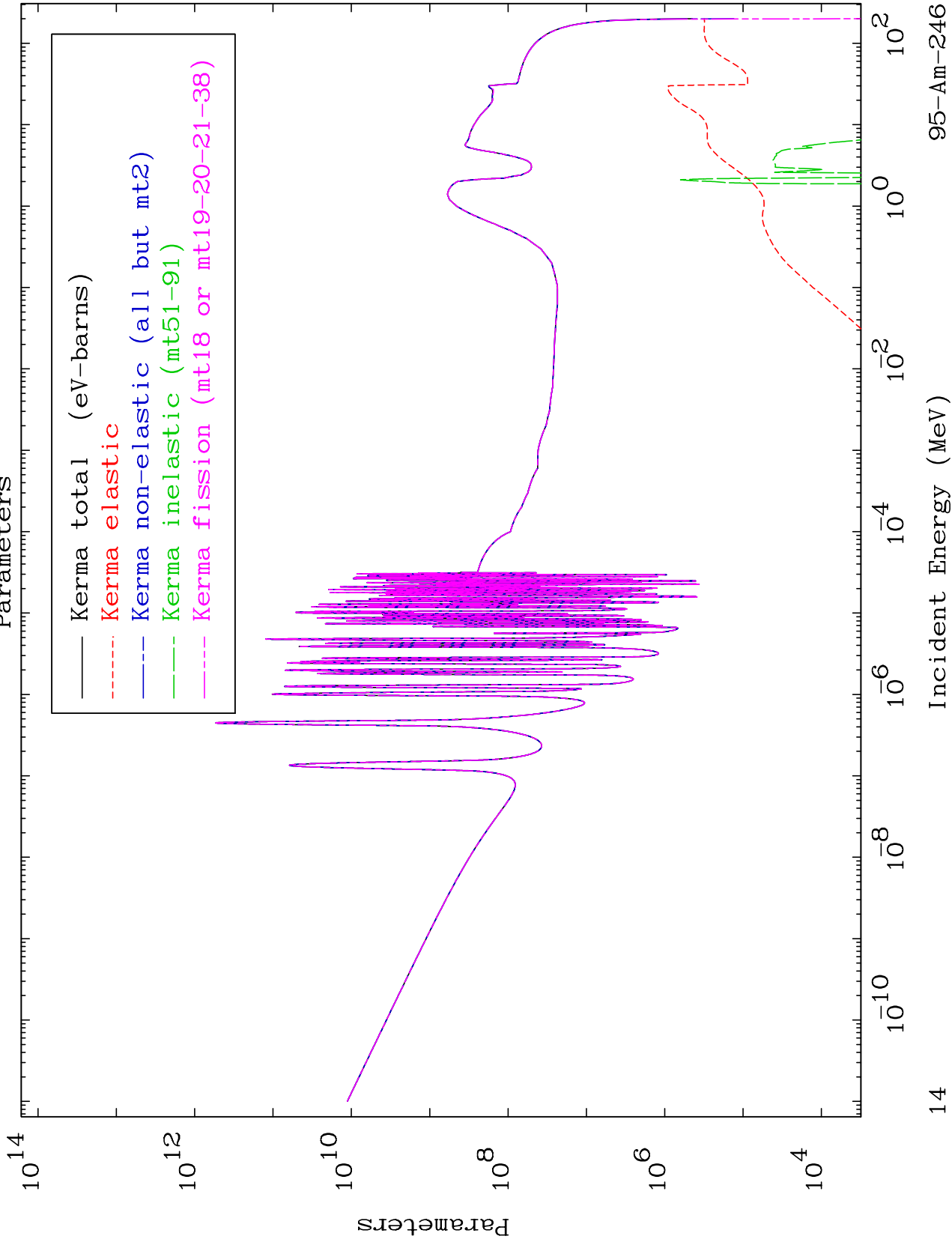
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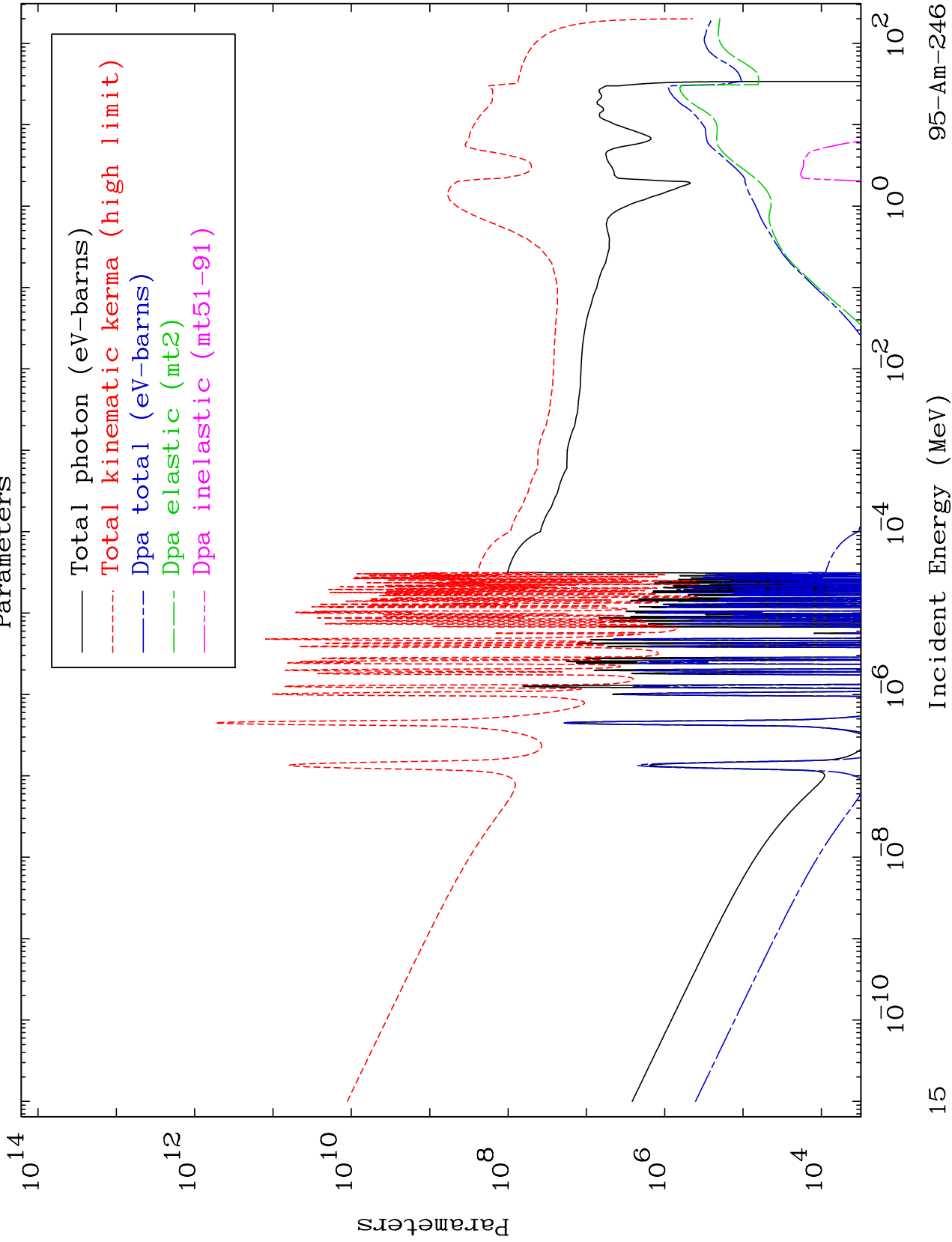


95-Am-246

Incident Energy (MeV)

13





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Elastic Legendre Coefficients

95-Am-246



16

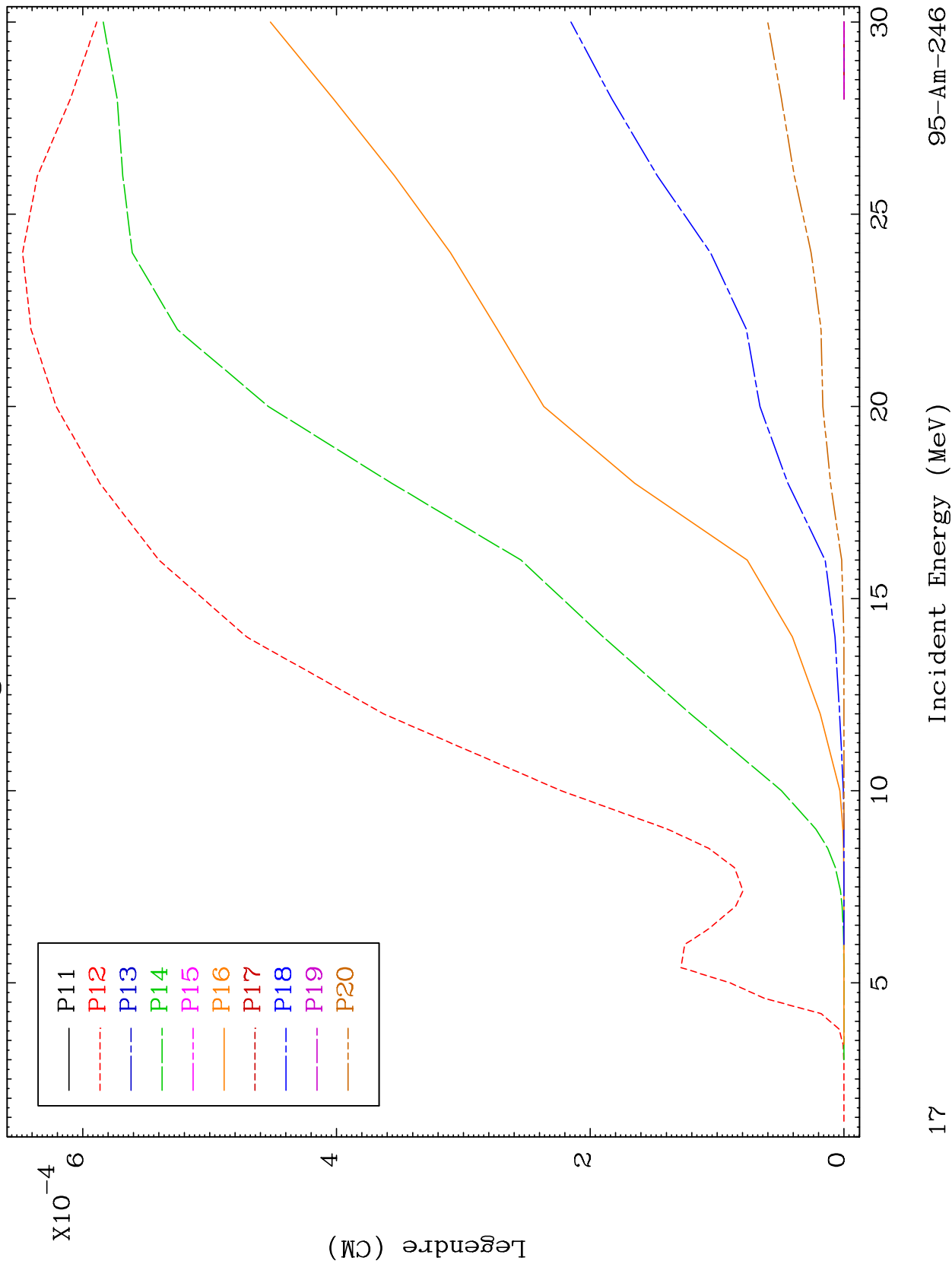
Incident Energy (MeV)

95-Am-246

MAT 9559

Elastic Legendre Coefficients

95-Am-246

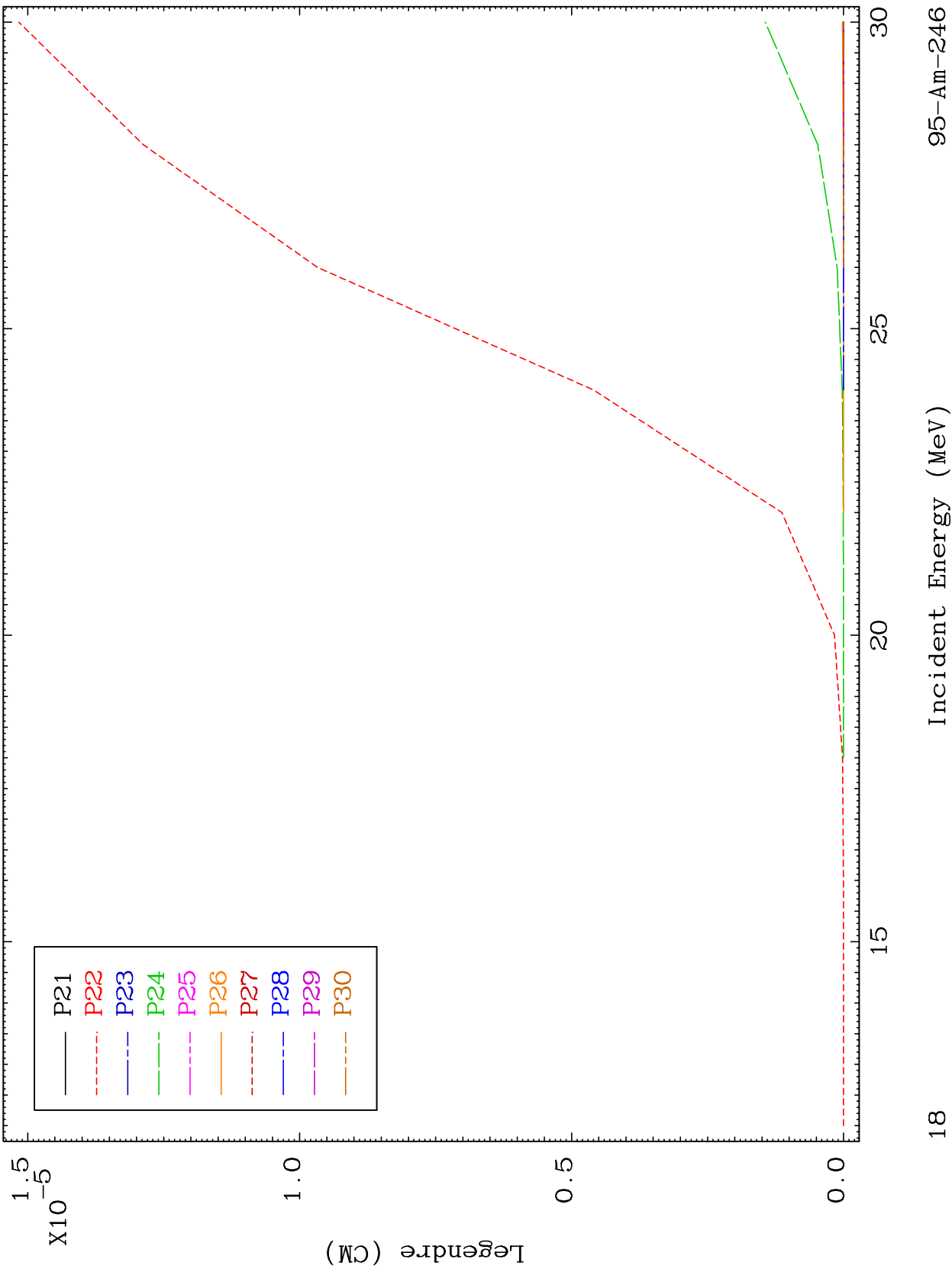


17

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Elastic
Legendre Coefficients

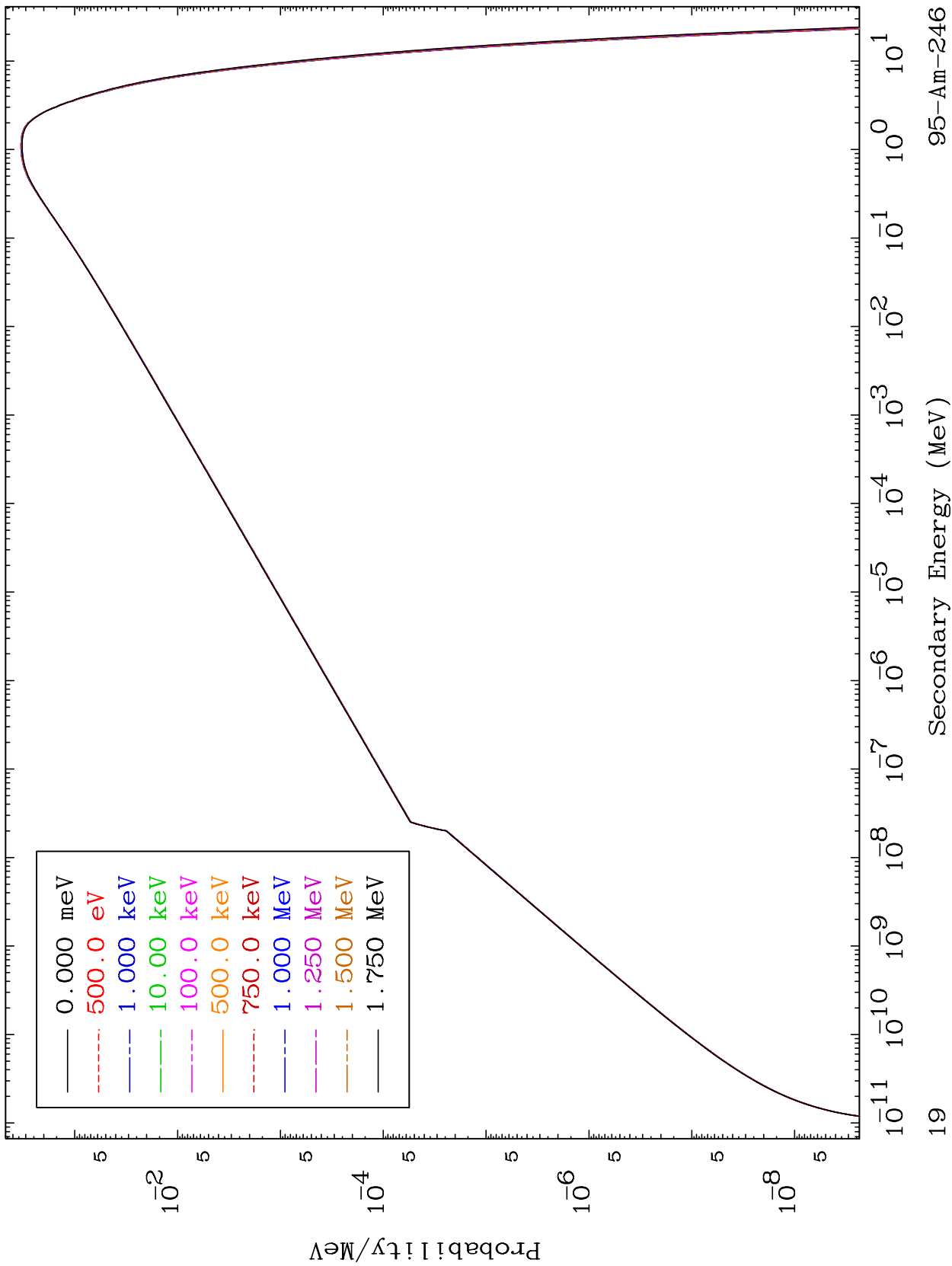
95-Am-246



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Fission Energy Distribution

95-Am-246



95-Am-246

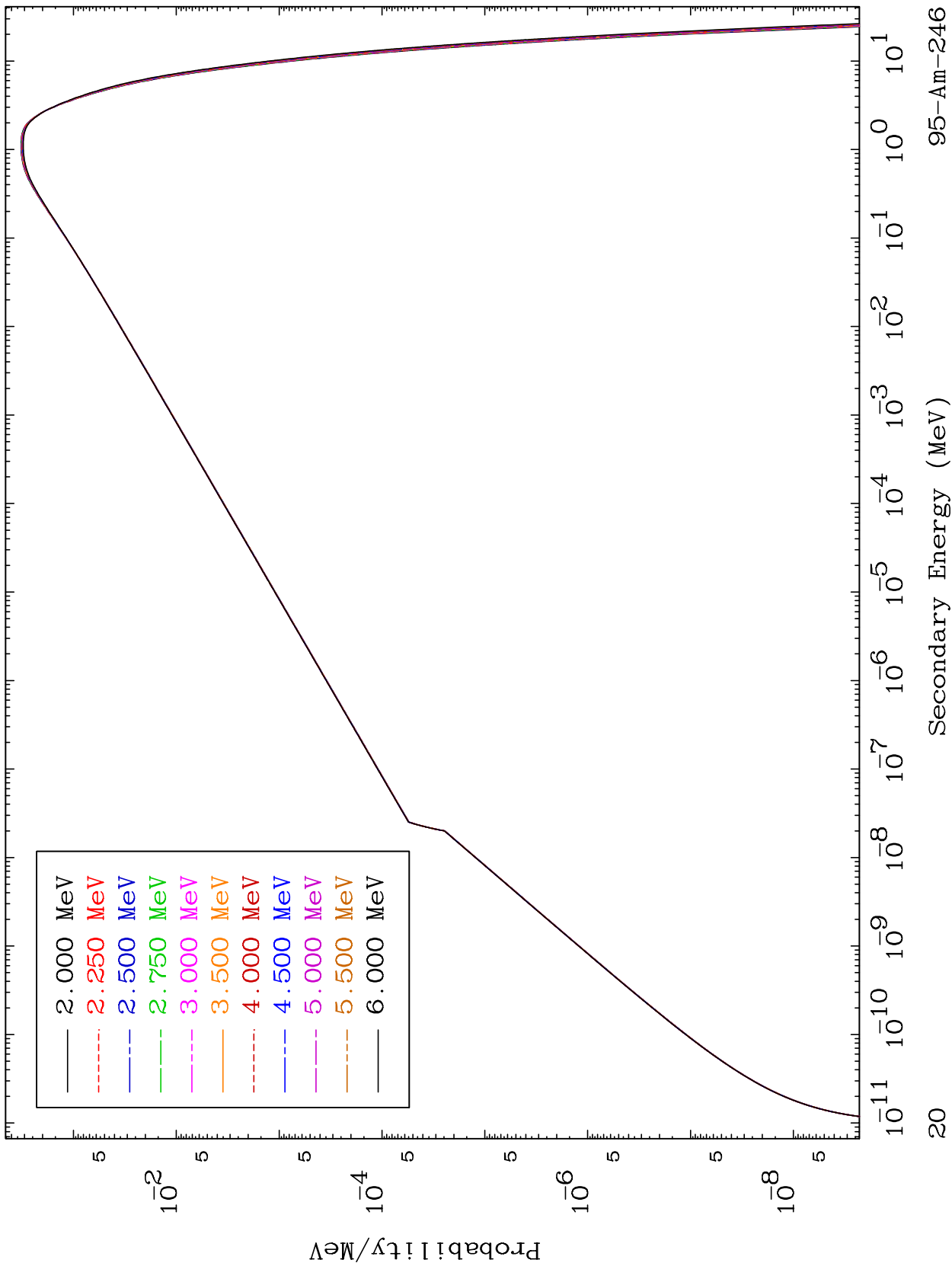
Secondary Energy (MeV)

19

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Fission Energy Distribution

95-Am-246



95-Am-246

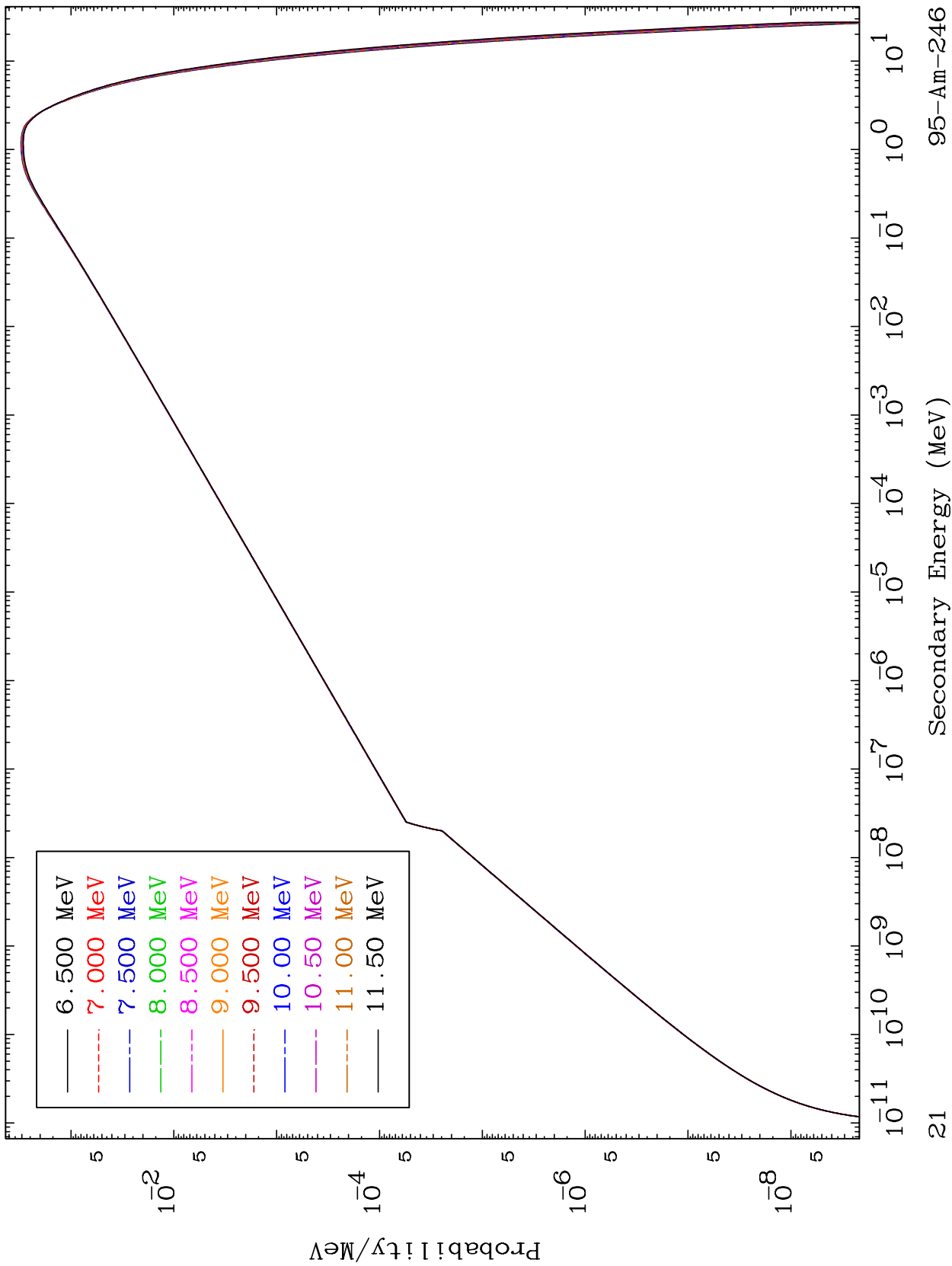
Secondary Energy (MeV)

20

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Fission Energy Distribution

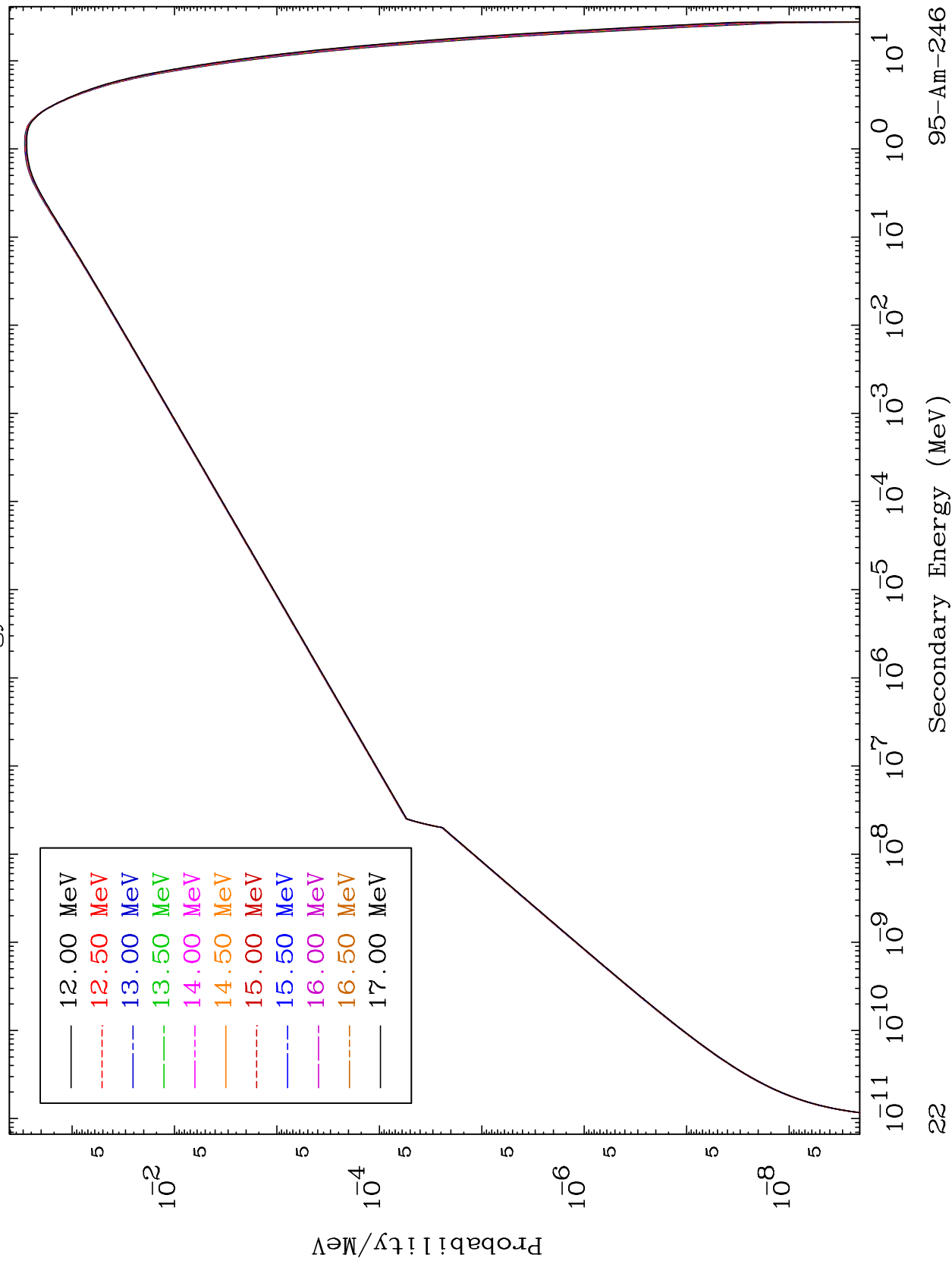
95-Am-246



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Fission Energy Distribution

95-Am-246



95-Am-246

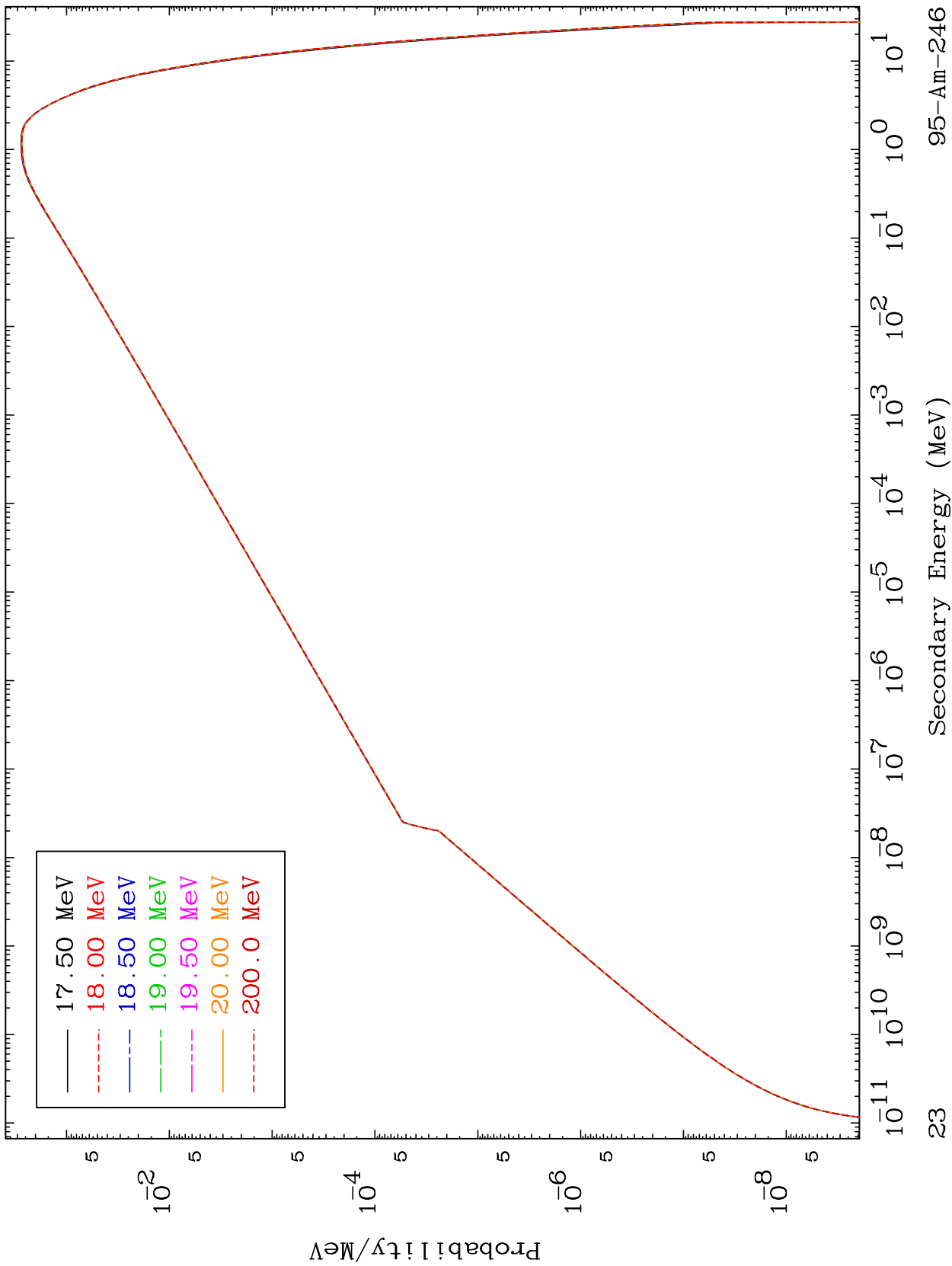
Secondary Energy (MeV)

22

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Fission Energy Distribution

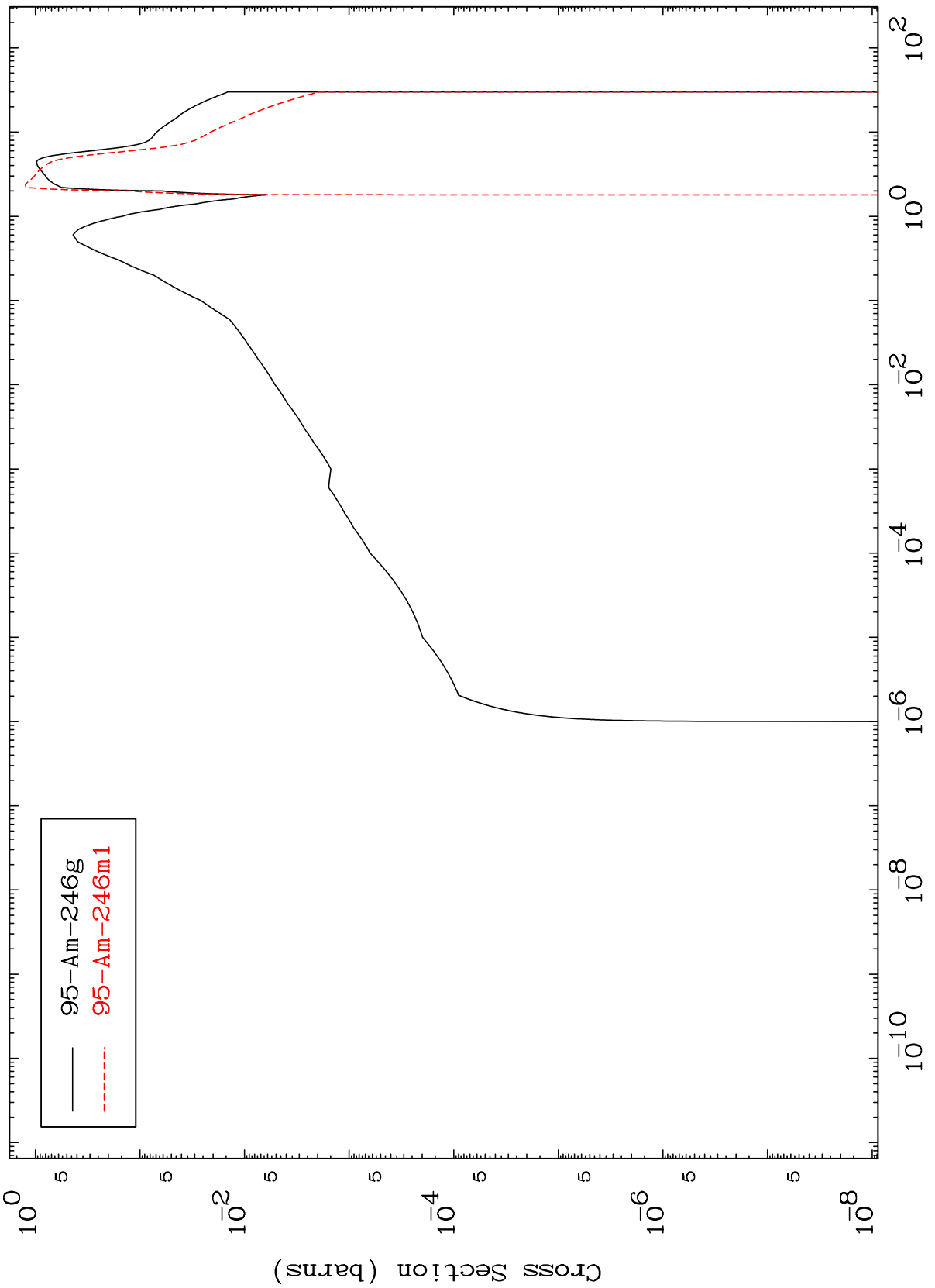
95-Am-246



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Inelastic
Radionuclide Production Cross Section

95-Am-246



95-Am-246g
95-Am-246m1

Incident Energy (MeV)

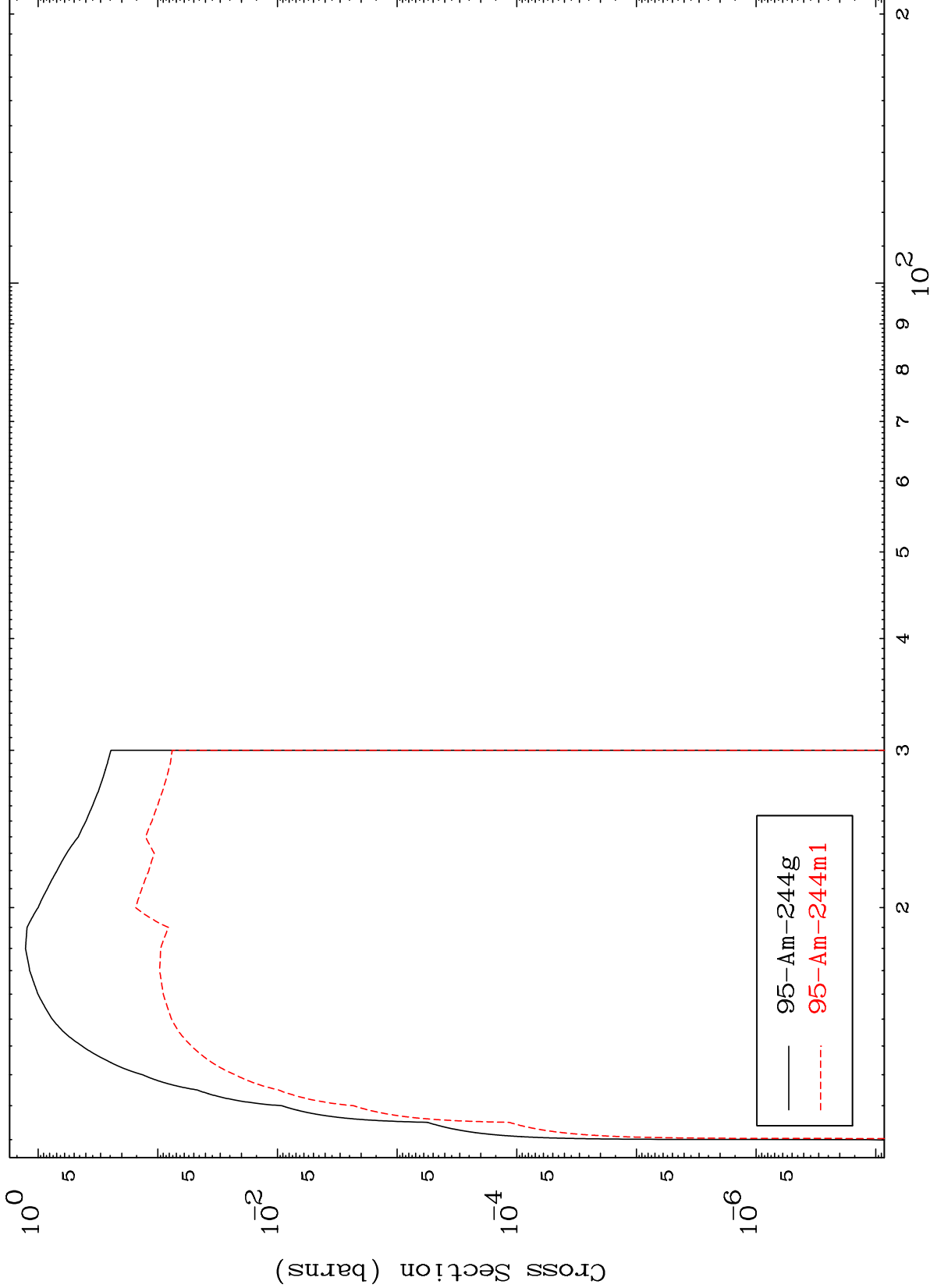
95-Am-246

24

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95-Am-246

(n,3n)
Radionuclide Production Cross Section



25

95-Am-246

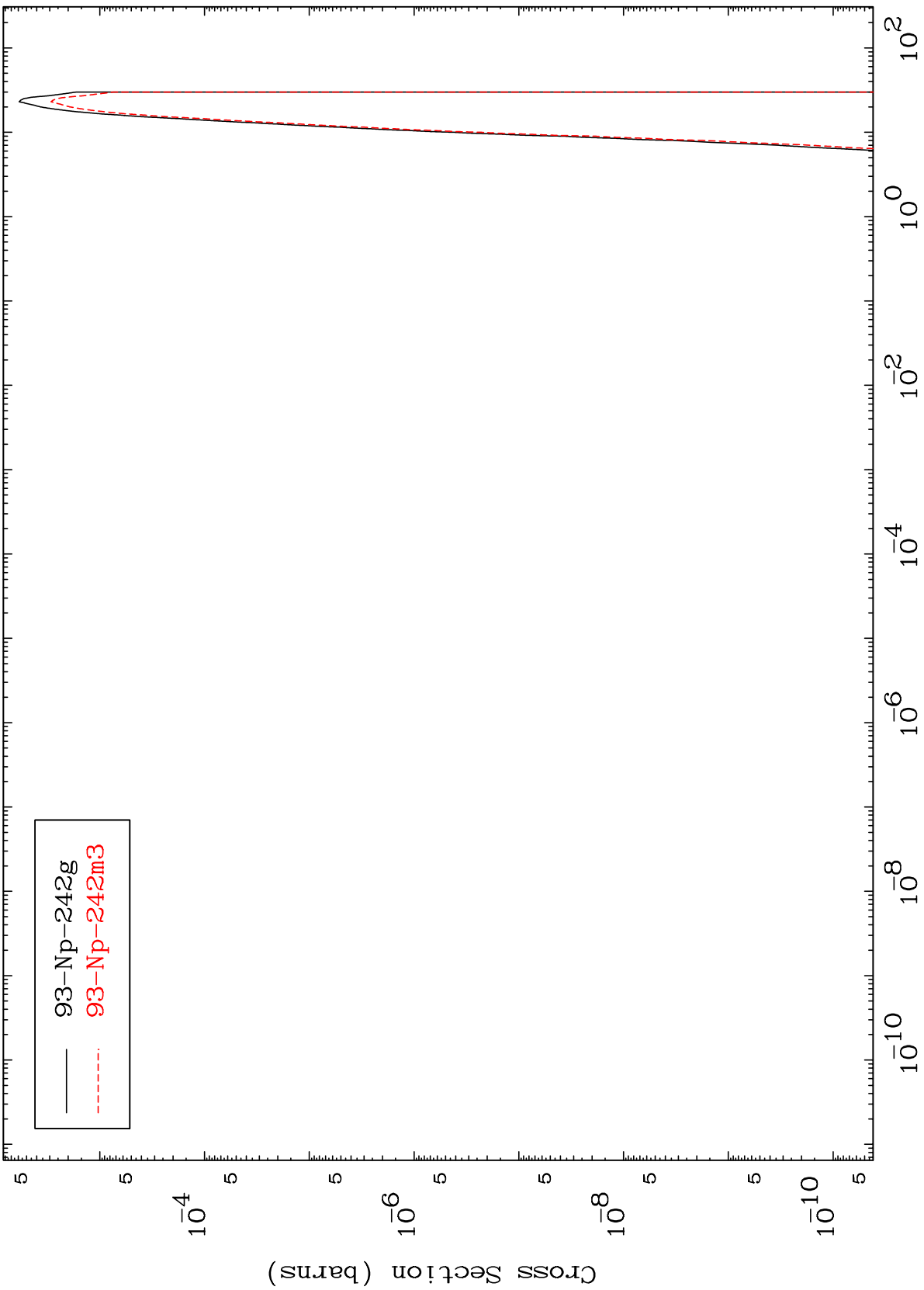
Incident Energy (MeV)

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$(n, n') \alpha$

95-Am-246

Radionuclide Production Cross Section



26

Incident Energy (MeV)

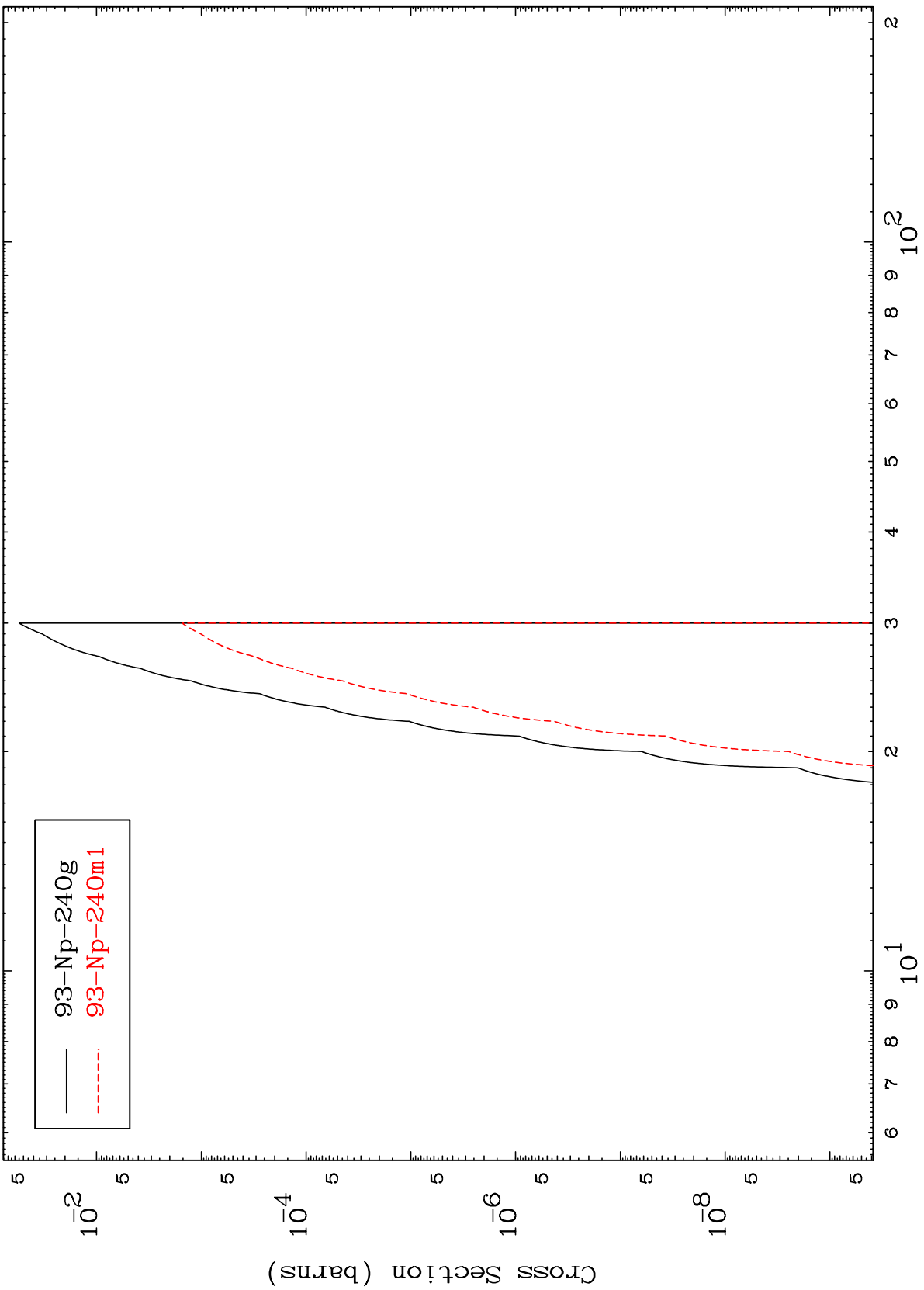
95-Am-246

MAT 9559

(n,3n) α

95-Am-246

Radionuclide Production Cross Section



— 93-Np-240g
- - - 93-Np-240m1

27

Incident Energy (MeV)

95-Am-246