

Program EVALPLOT
(Version 2018-1)

by

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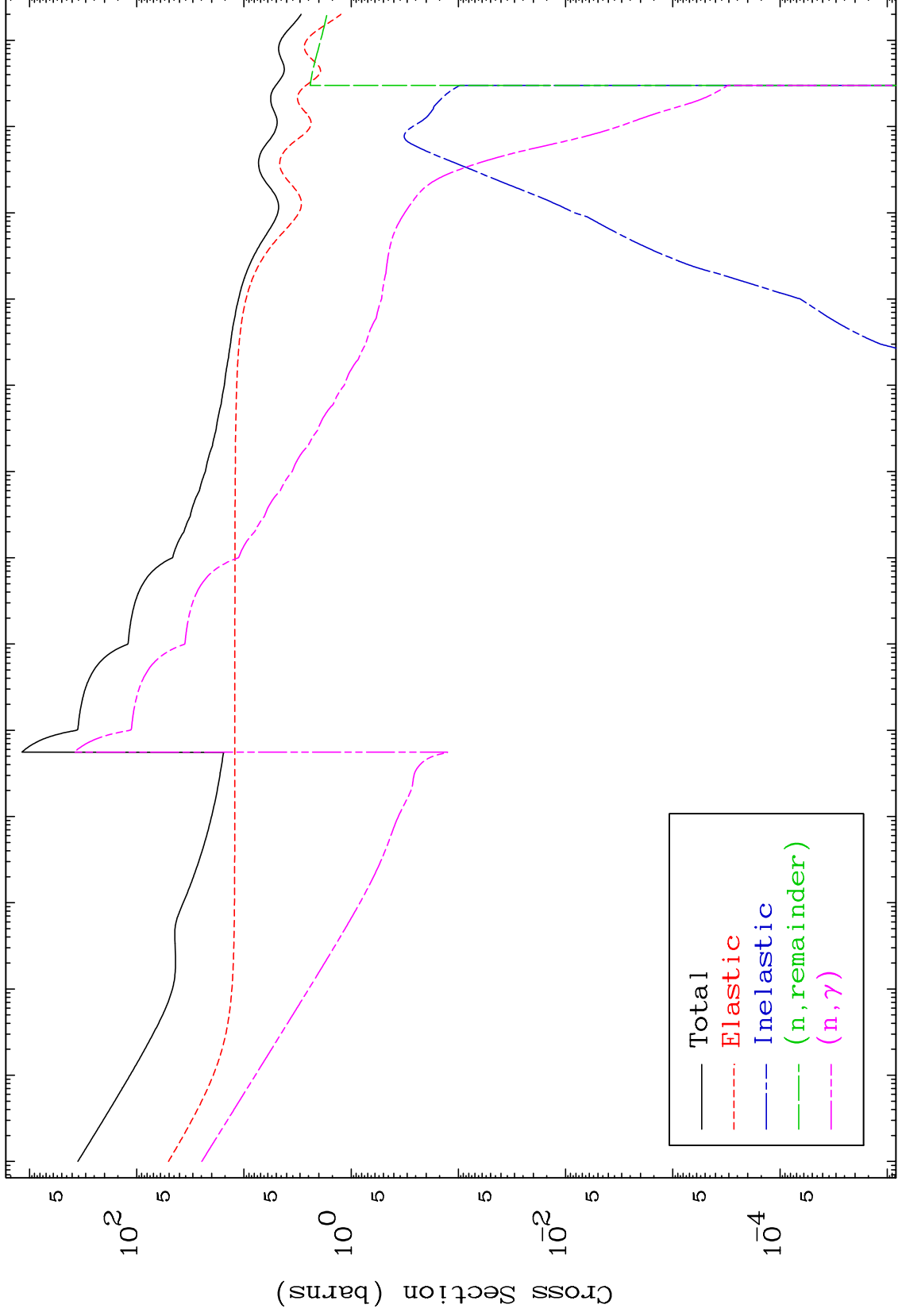
Press Mouse Button to Start

MAT 8511

Major

293 Kelvin Cross Sections

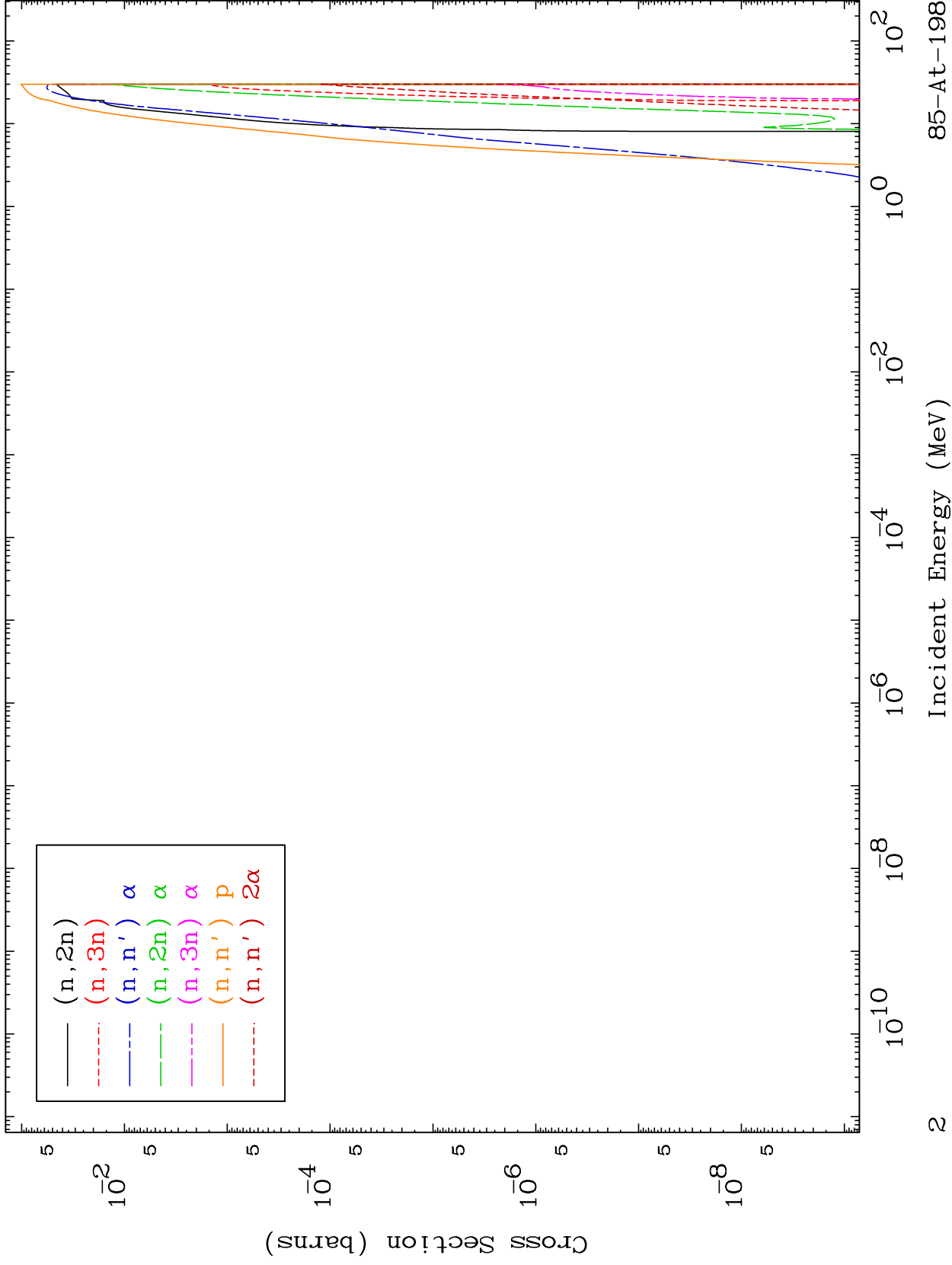
85-At-198



MAT 8511

Neutron Production
293 Kelvin Cross Sections

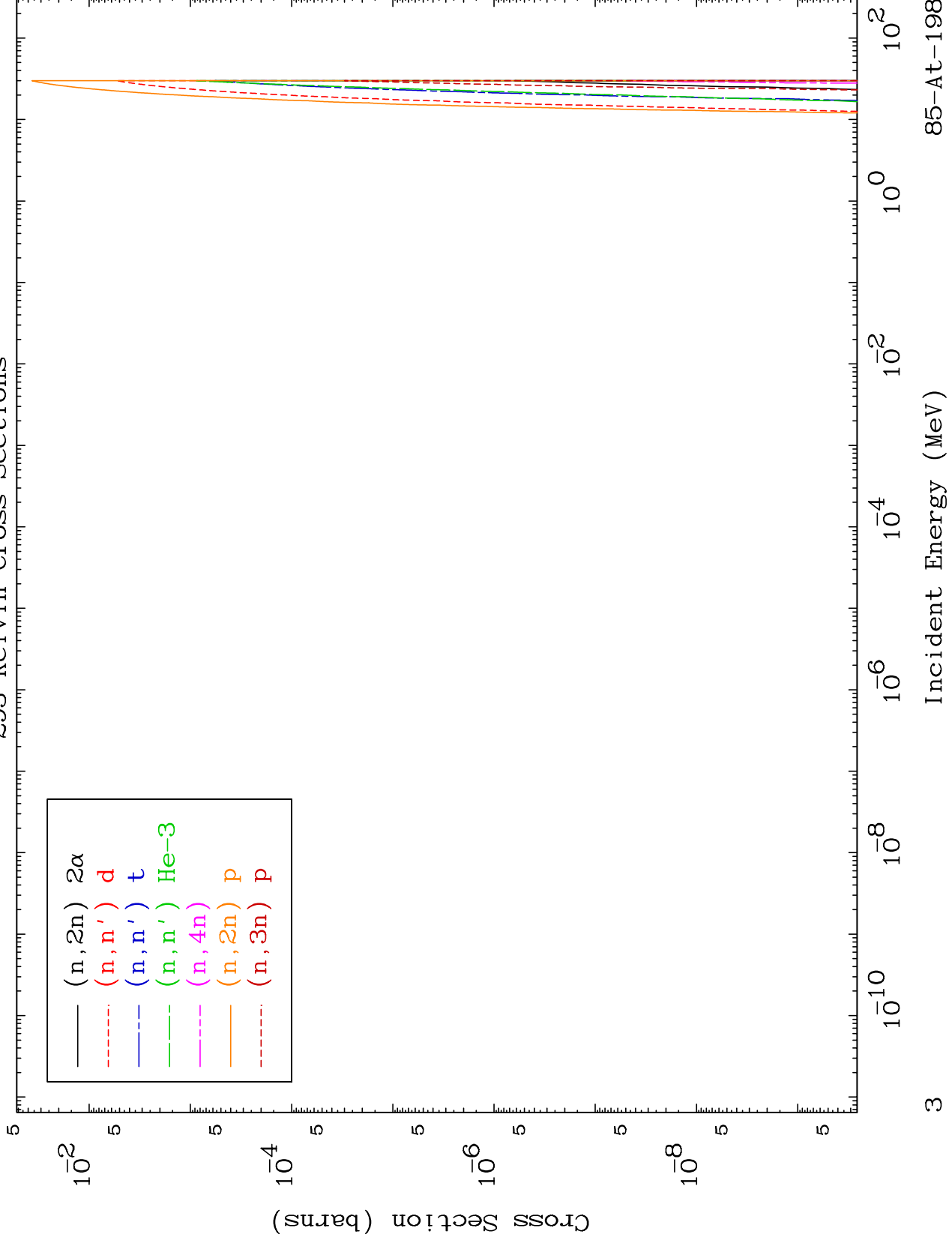
85-At-198



MAT 8511

Neutron Production
293 Kelvin Cross Sections

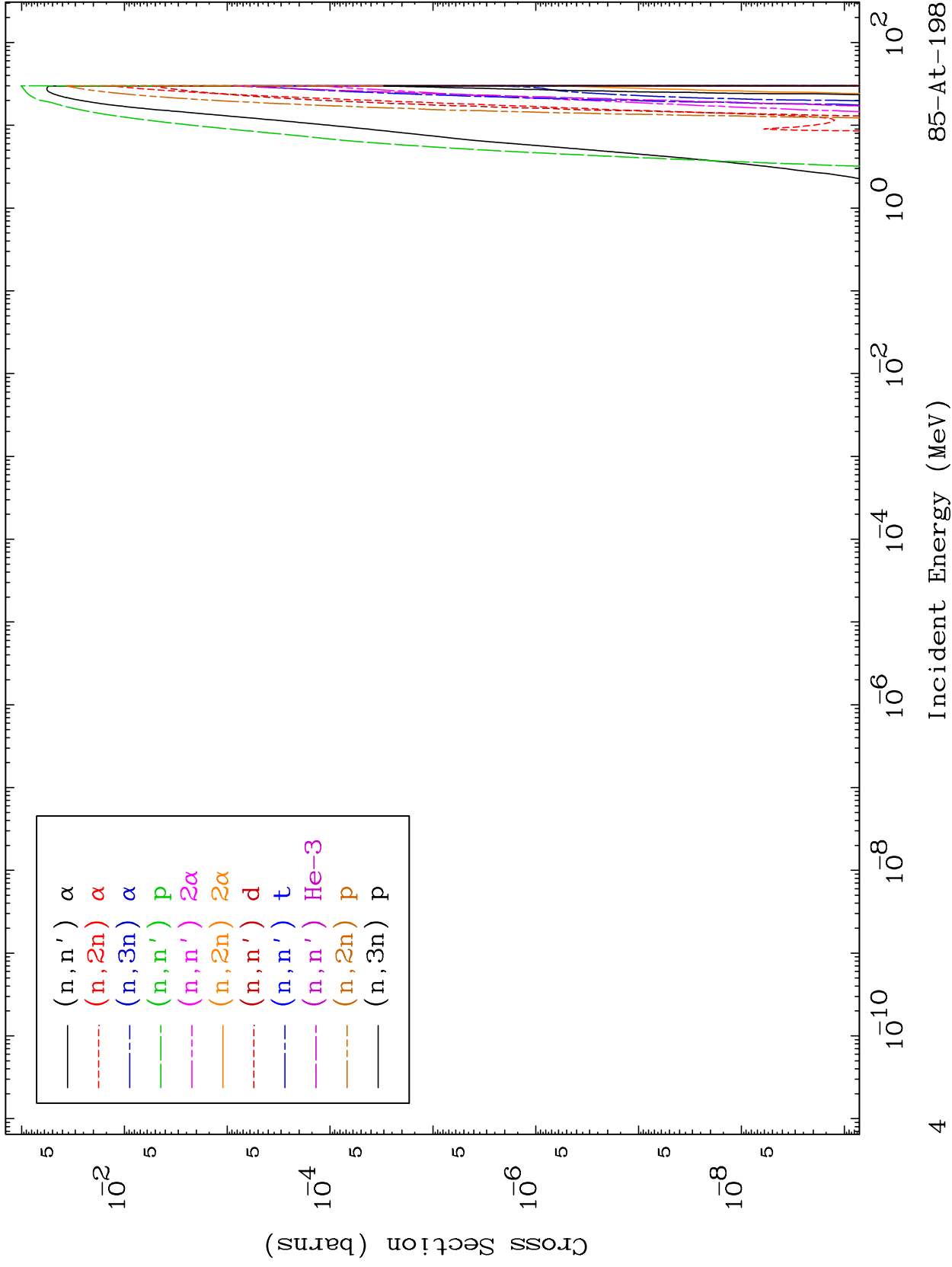
85-At-198

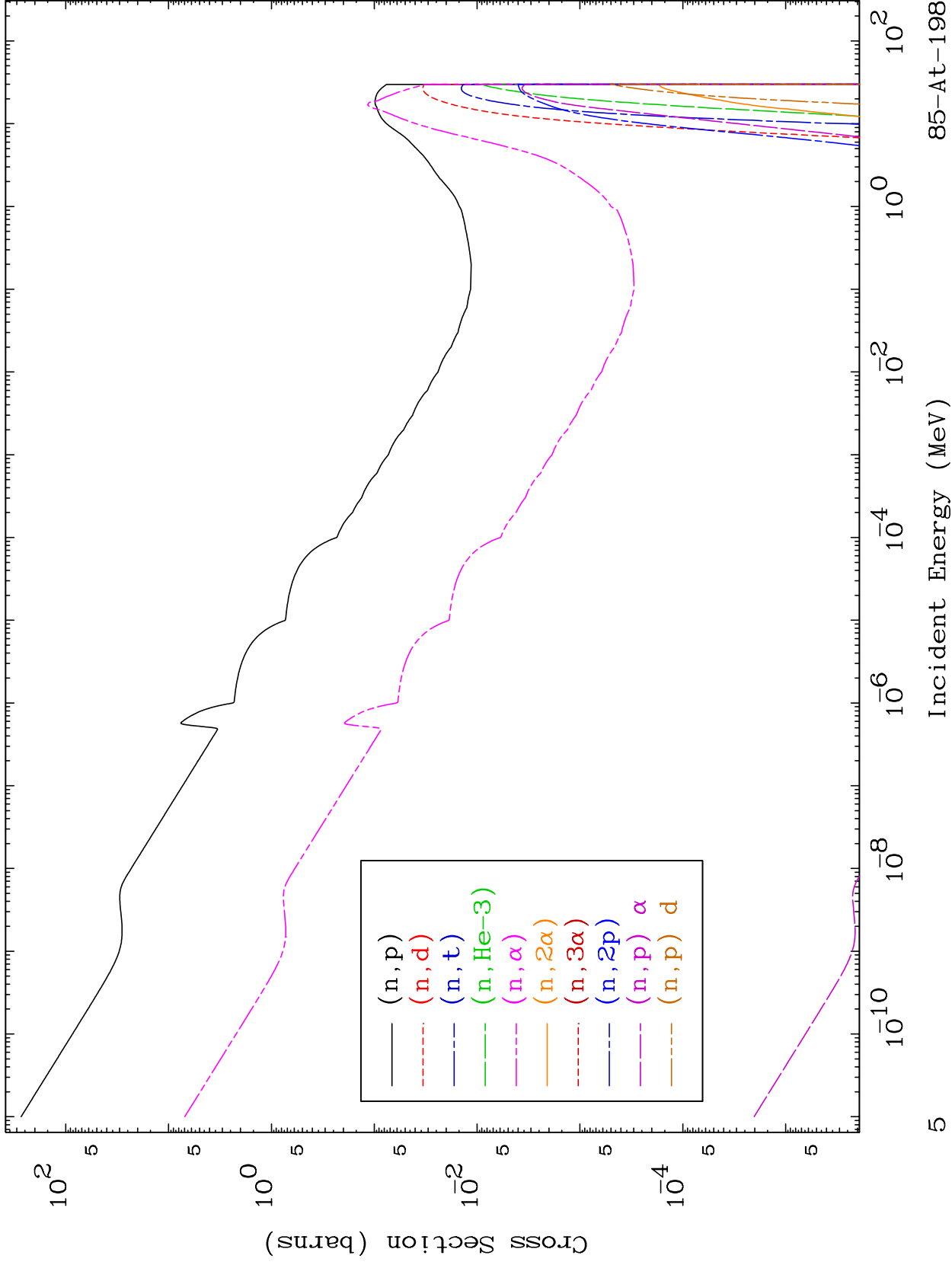


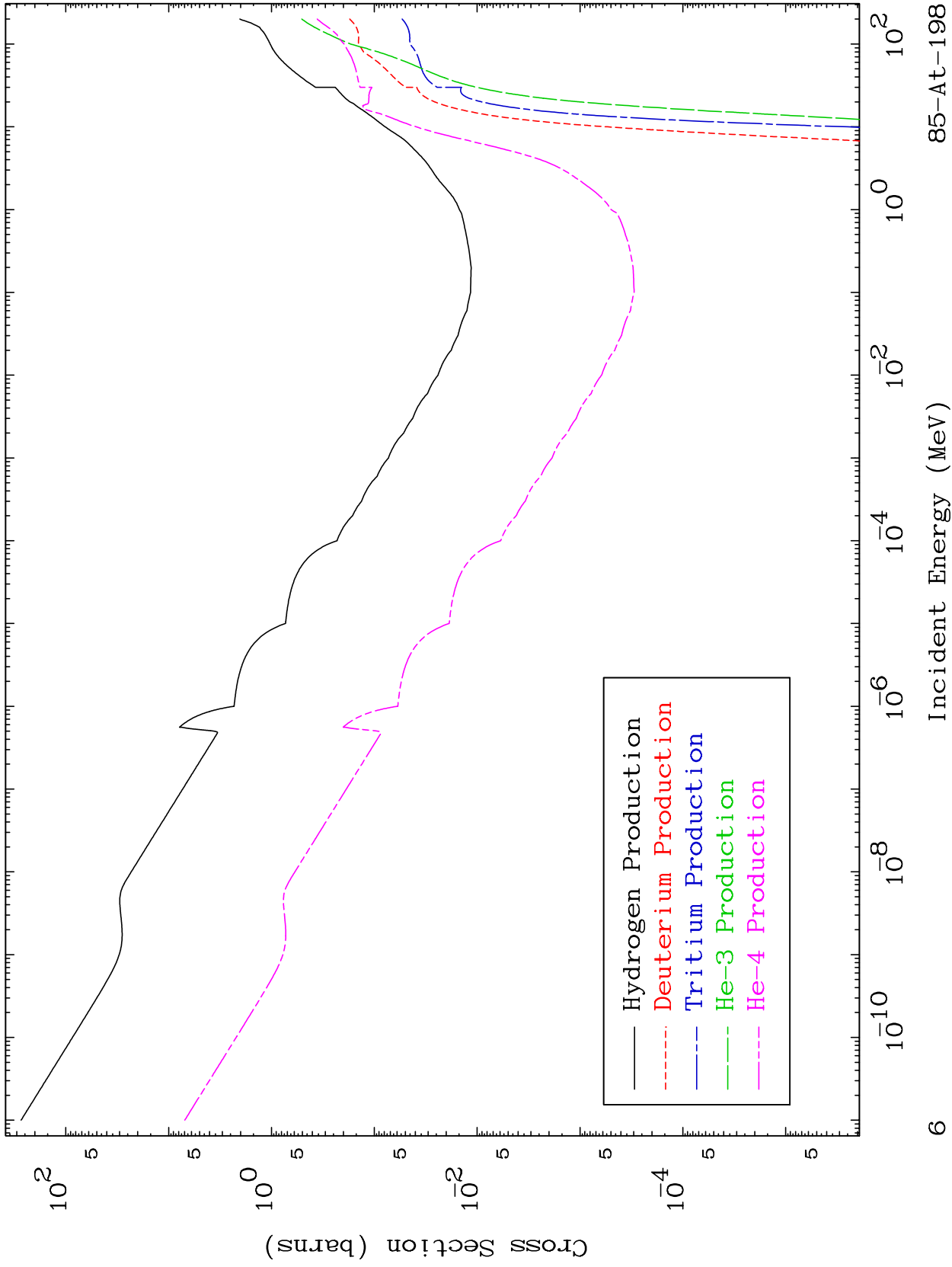
MAT 8511

Charged Particle
293 Kelvin Cross Sections

85-At-198





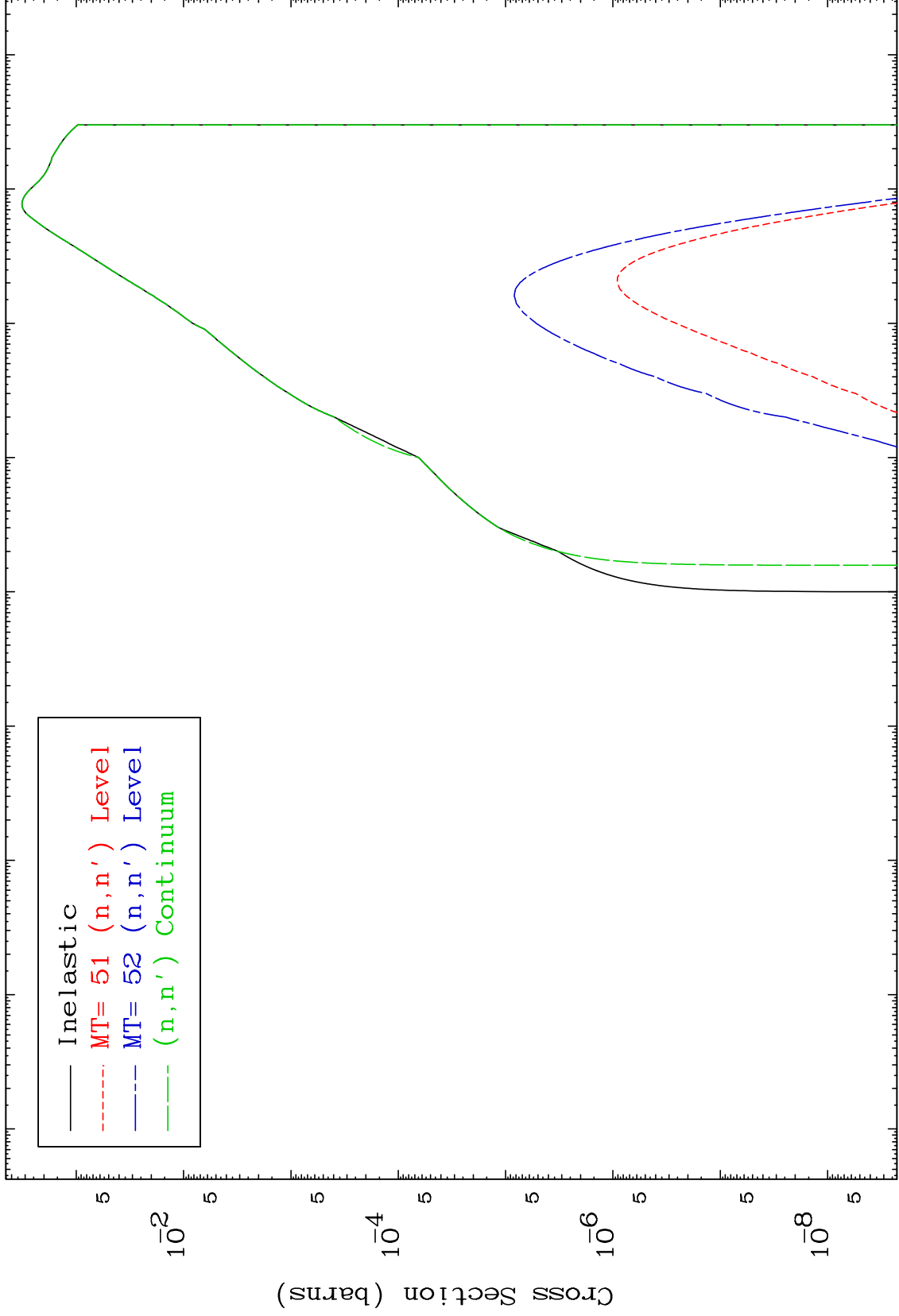


MAT 8511

(n,n') Level

85-At-198

293 Kelvin Cross Sections

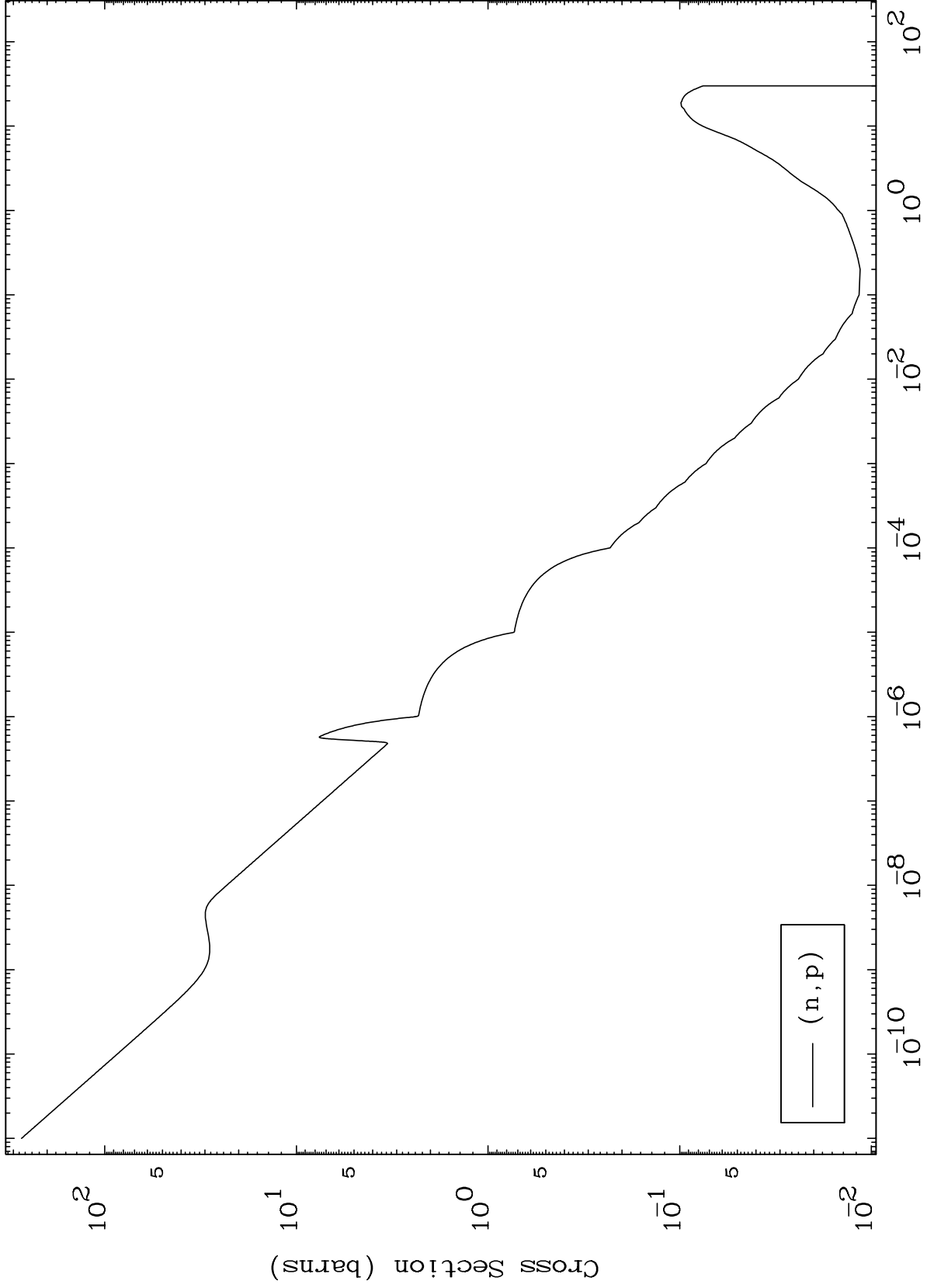


— Inelastic
- - - MT= 51 (n,n') Level
- - - MT= 52 (n,n') Level
- - - (n,n') Continuum

MAT 8511

(n,p) Levels
293 Kelvin Cross Sections

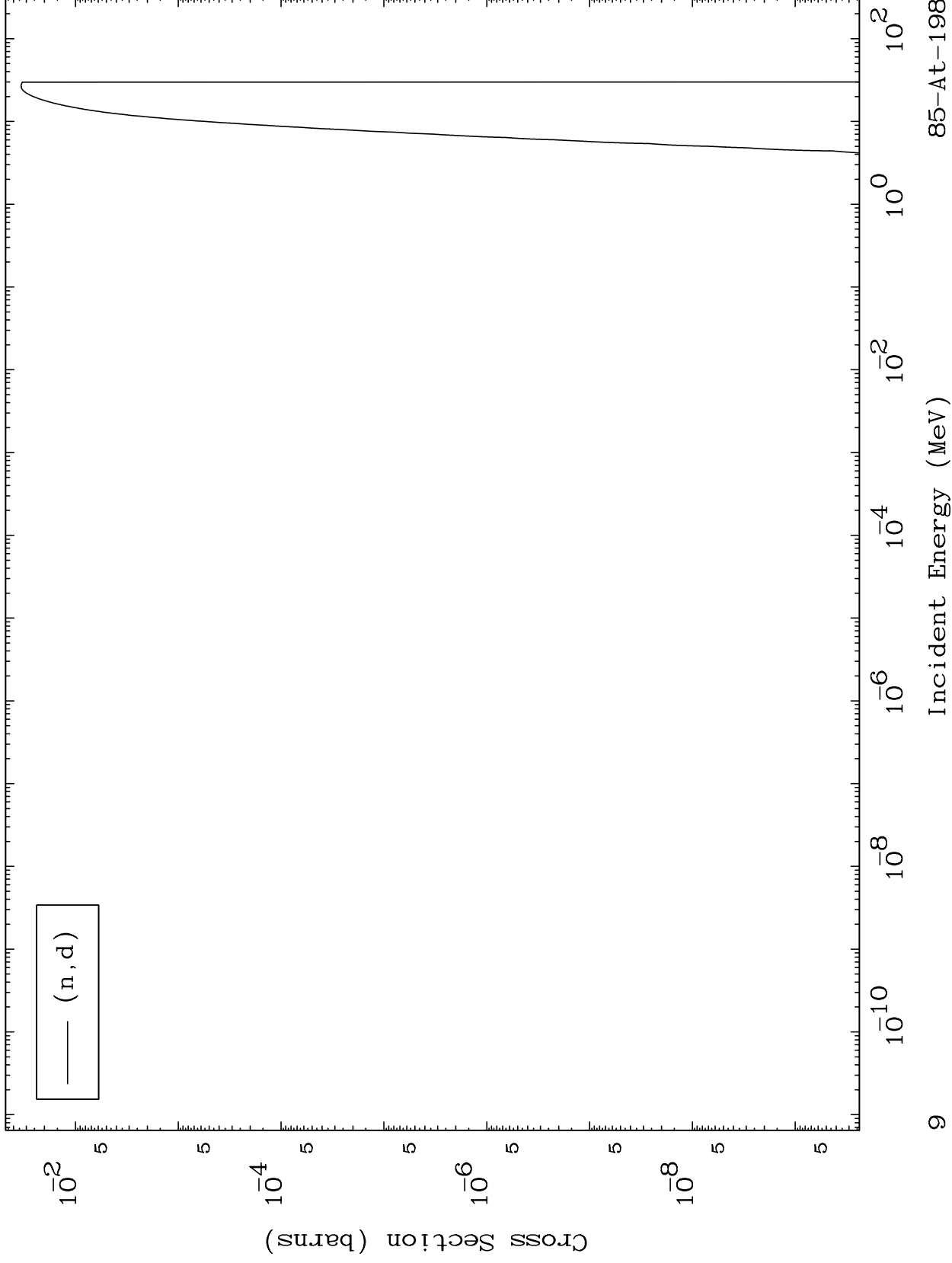
85-At-198



MAT 8511

(n,d) Levels
293 Kelvin Cross Sections

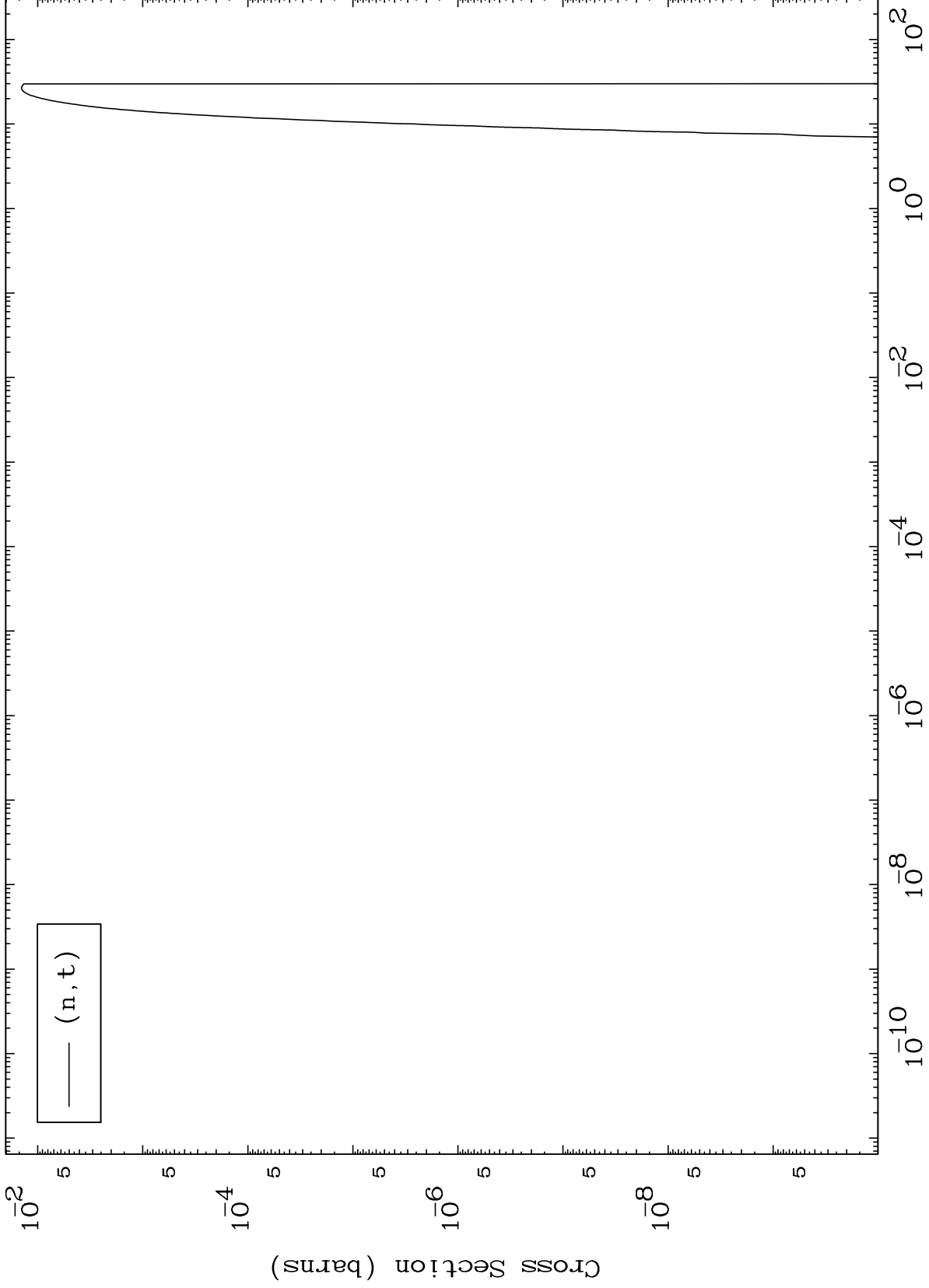
85-At-198



MAT 8511

(n,t) Levels
293 Kelvin Cross Sections

85-At-198



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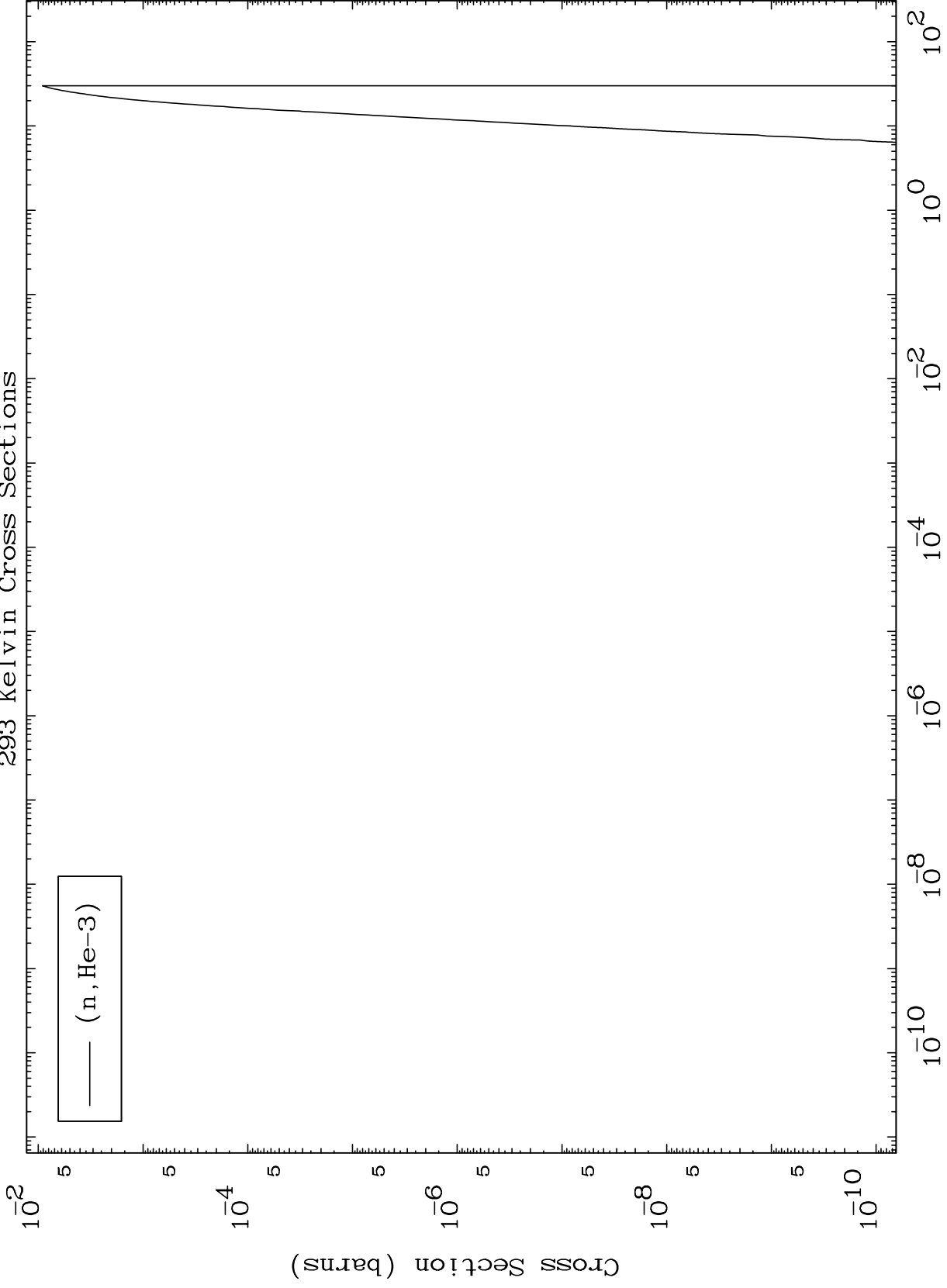
Incident Energy (MeV)

85-At-198

MAT 8511

(n,He3) Levels
293 Kelvin Cross Sections

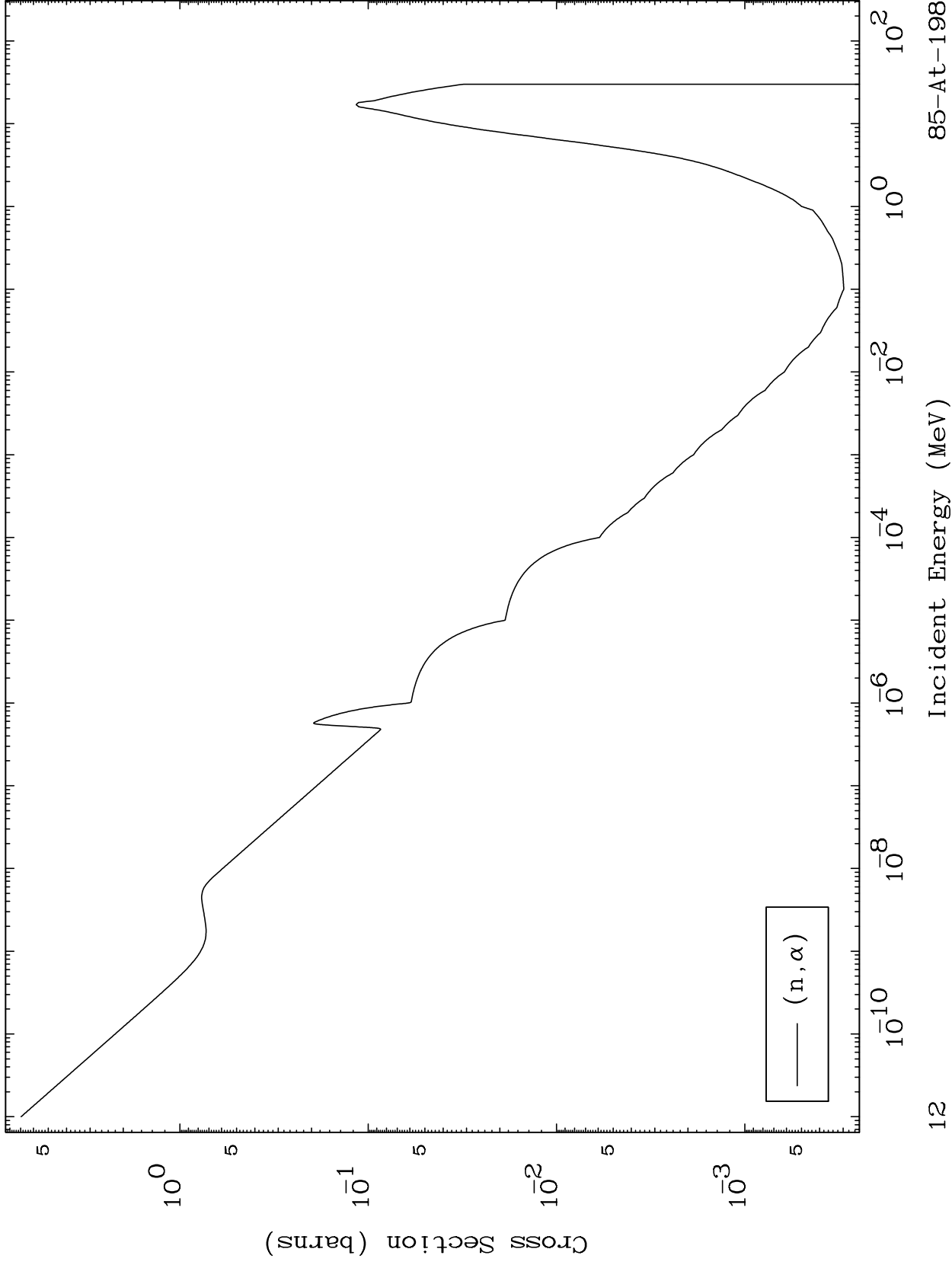
85-At-198

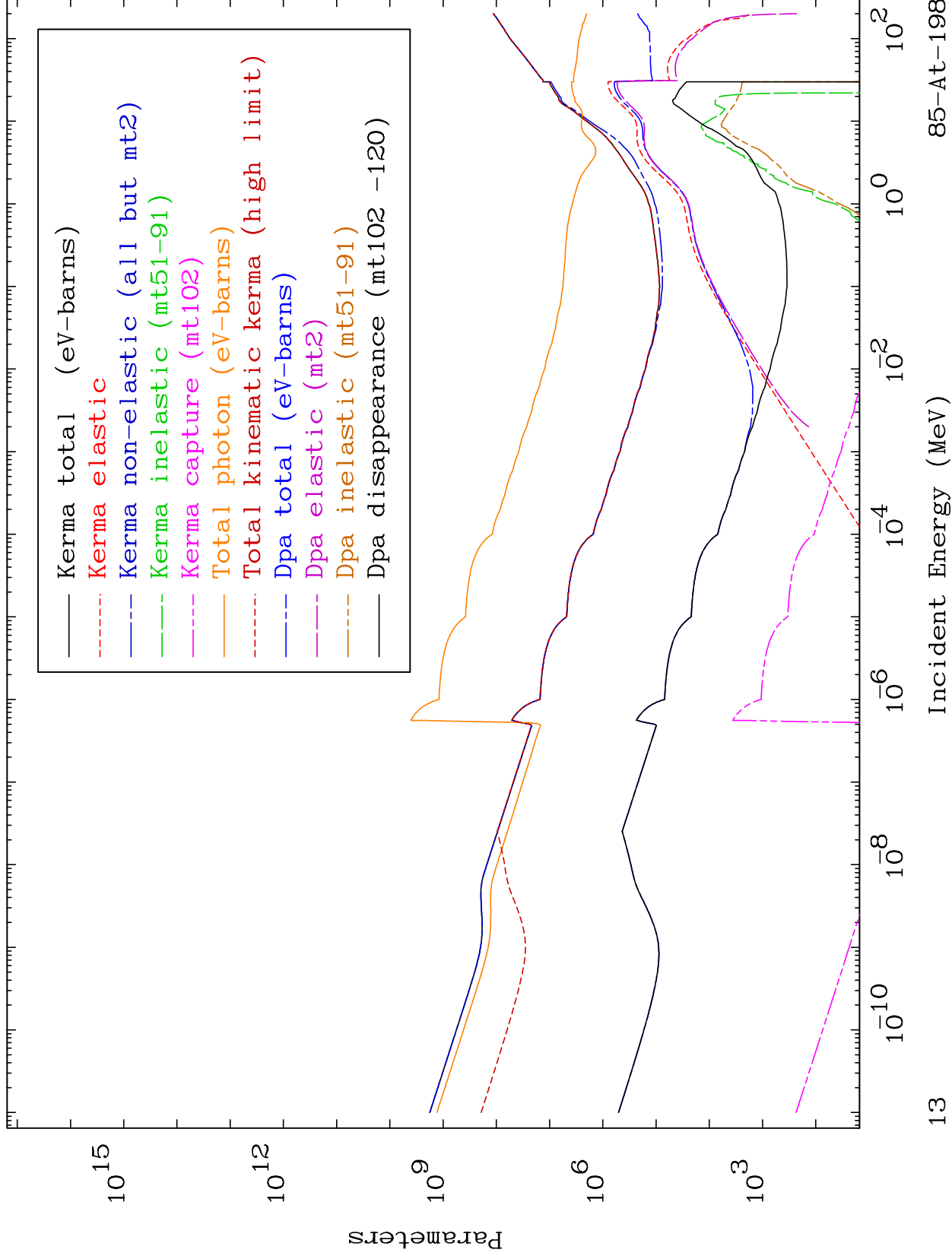


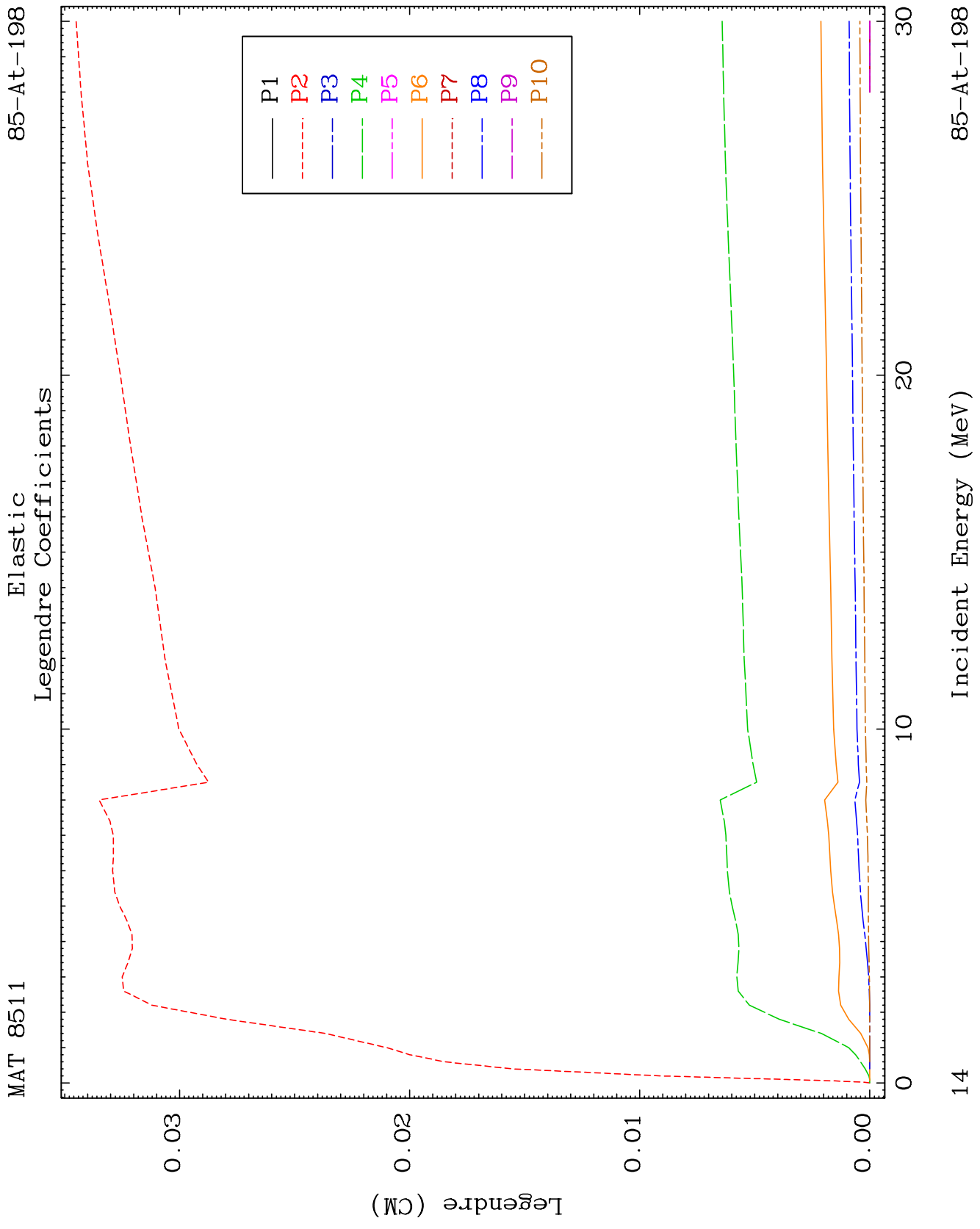
MAT 8511

(n, α) Levels
293 Kelvin Cross Sections

85-At-198



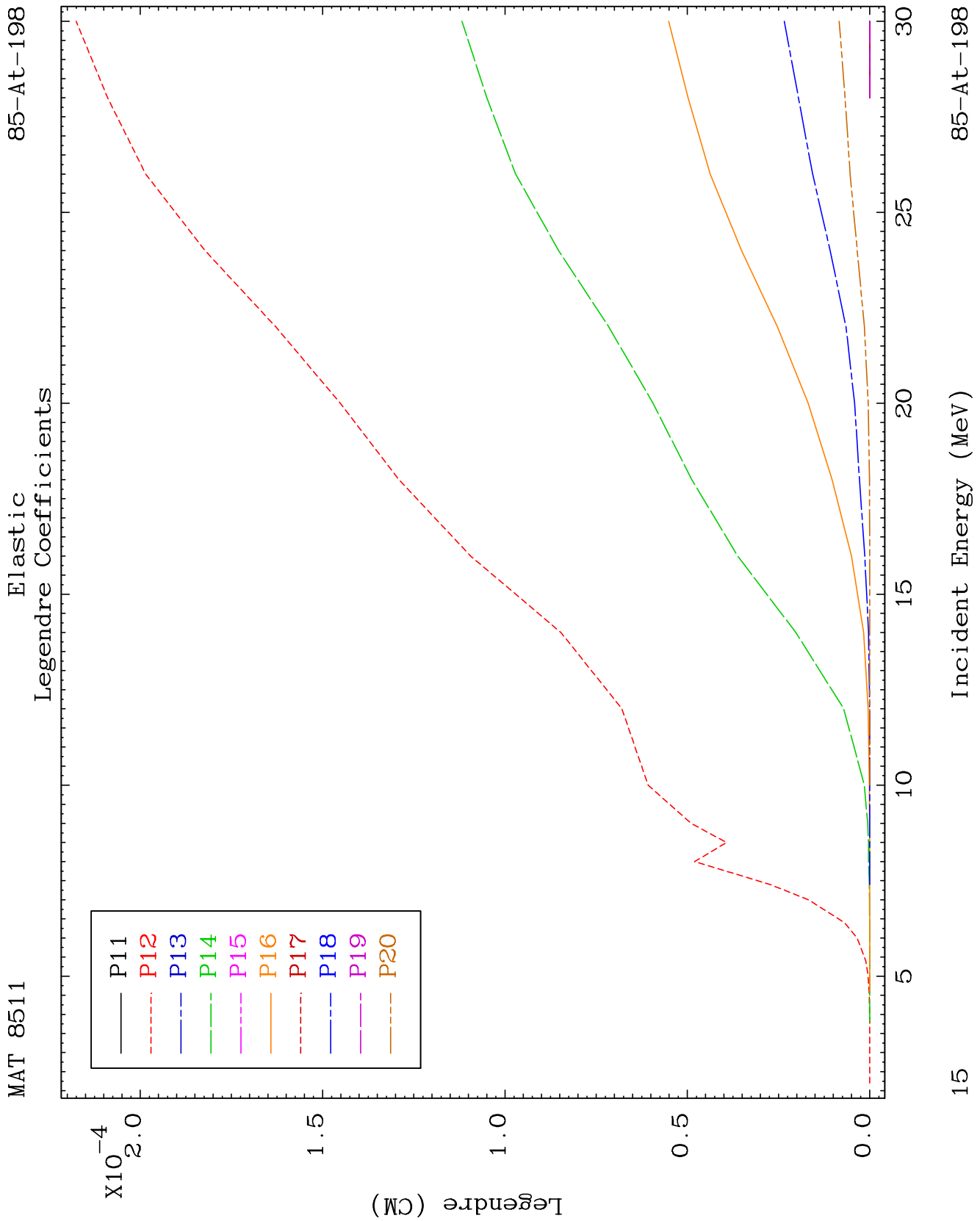




85-At-198

Incident Energy (MeV)

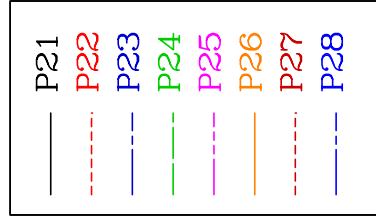
14



MAT 8511

Elastic
Legendre Coefficients

85-At-198



$\times 10^{-6}$

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

25

30

16

Incident Energy (MeV)

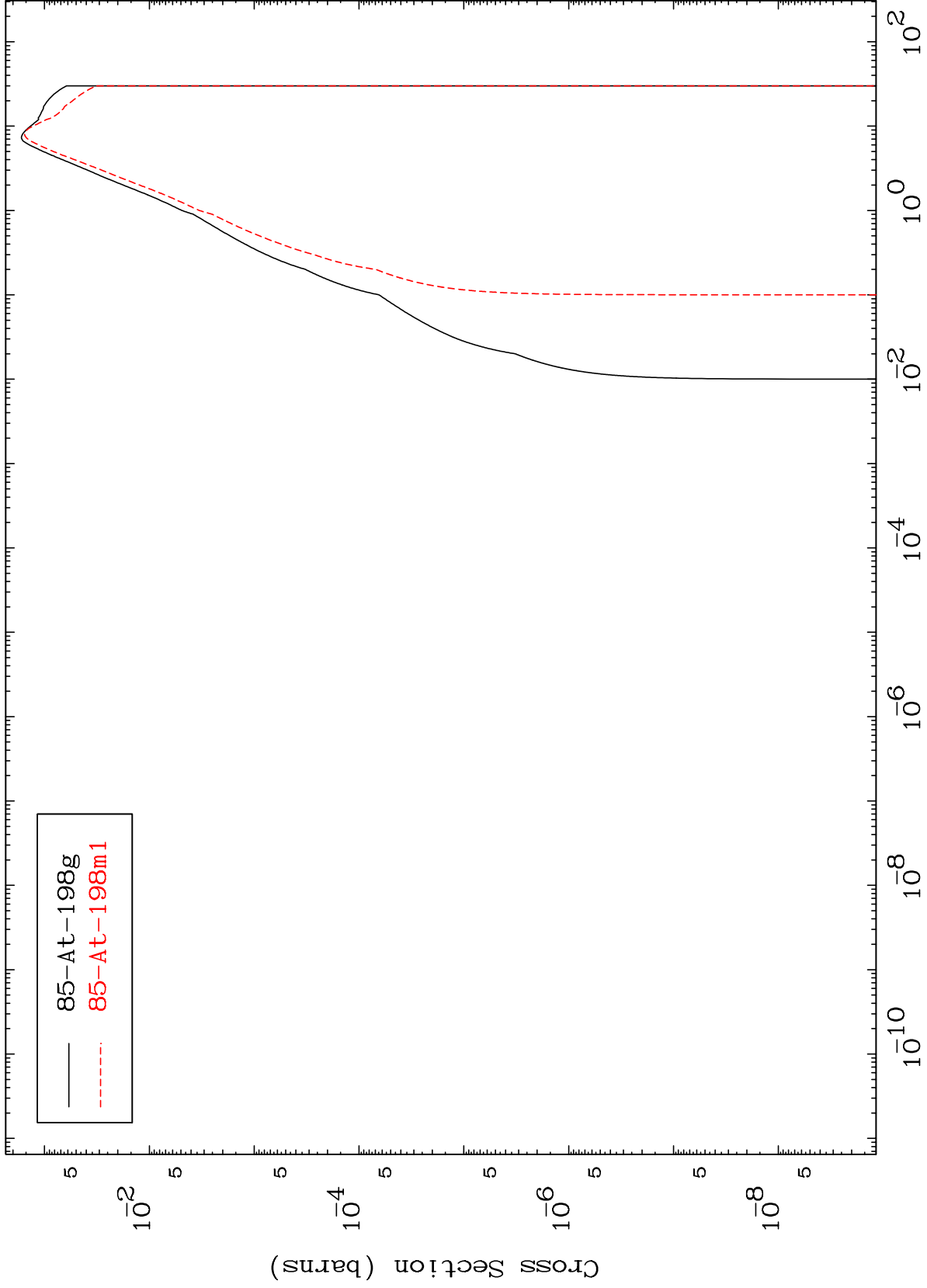
85-At-198



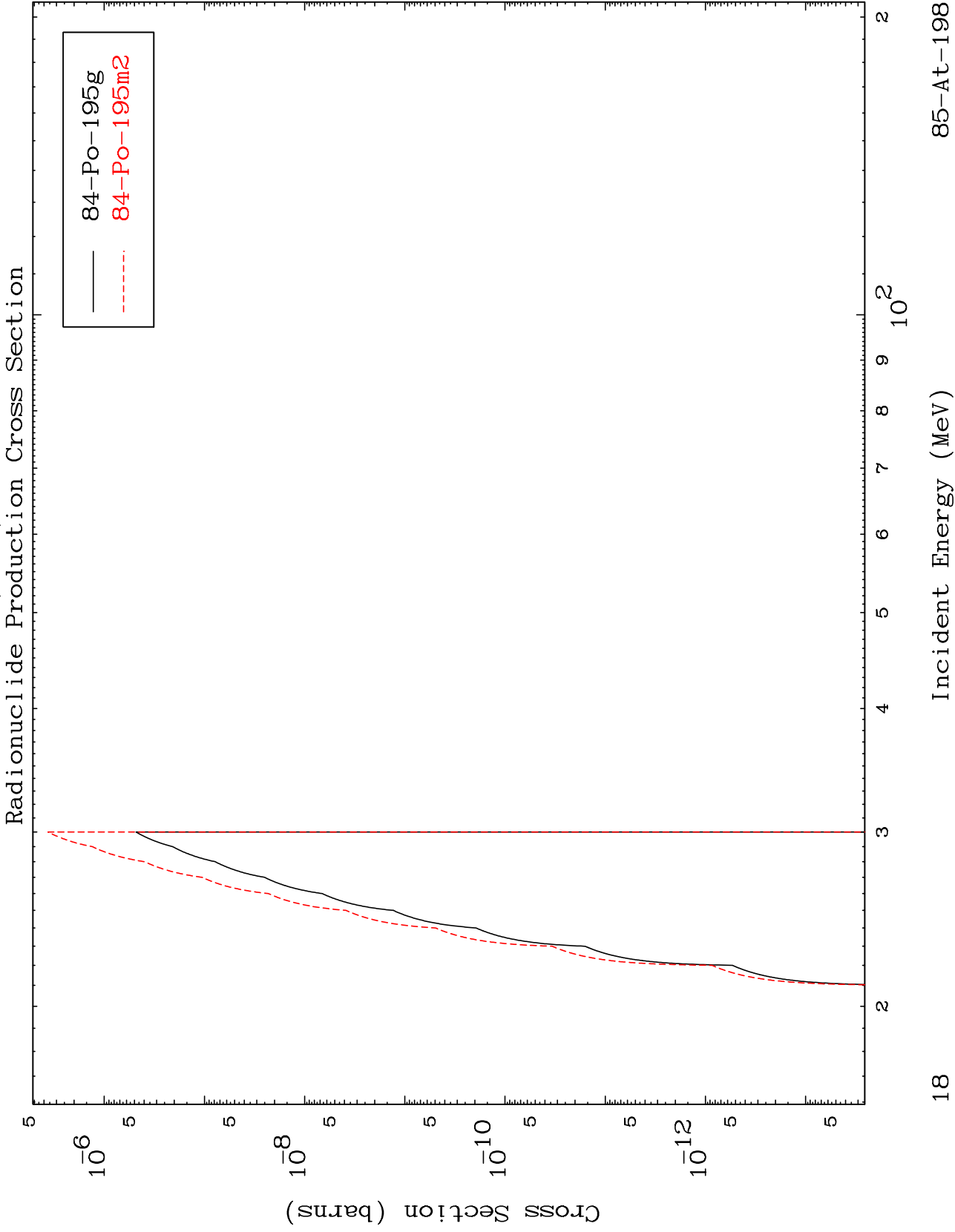
MAT 8511

Inelastic
Radionuclide Production Cross Section

85-At-198



85-At-198

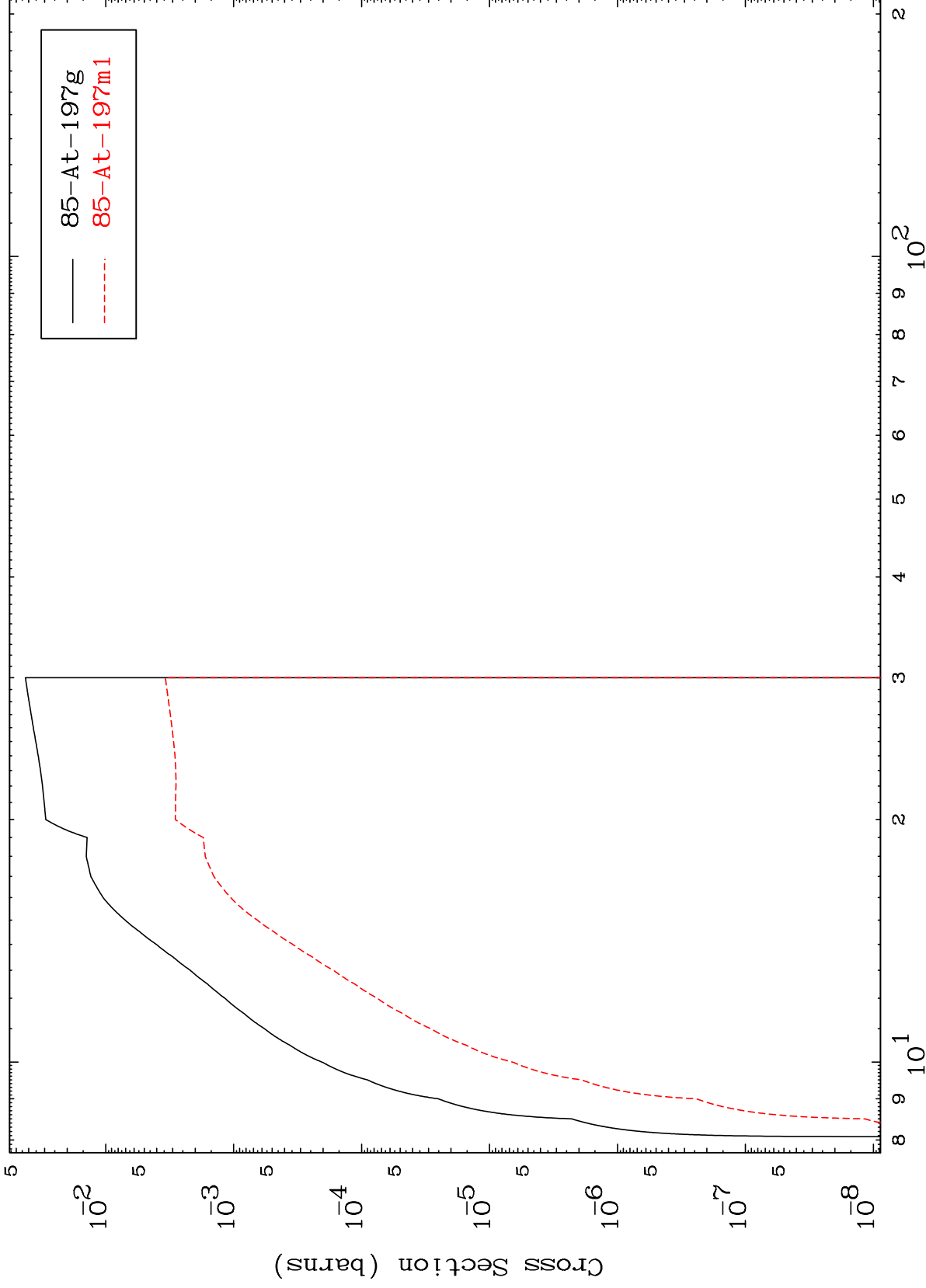


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(n,2n)

85-At-198

Radionuclide Production Cross Section

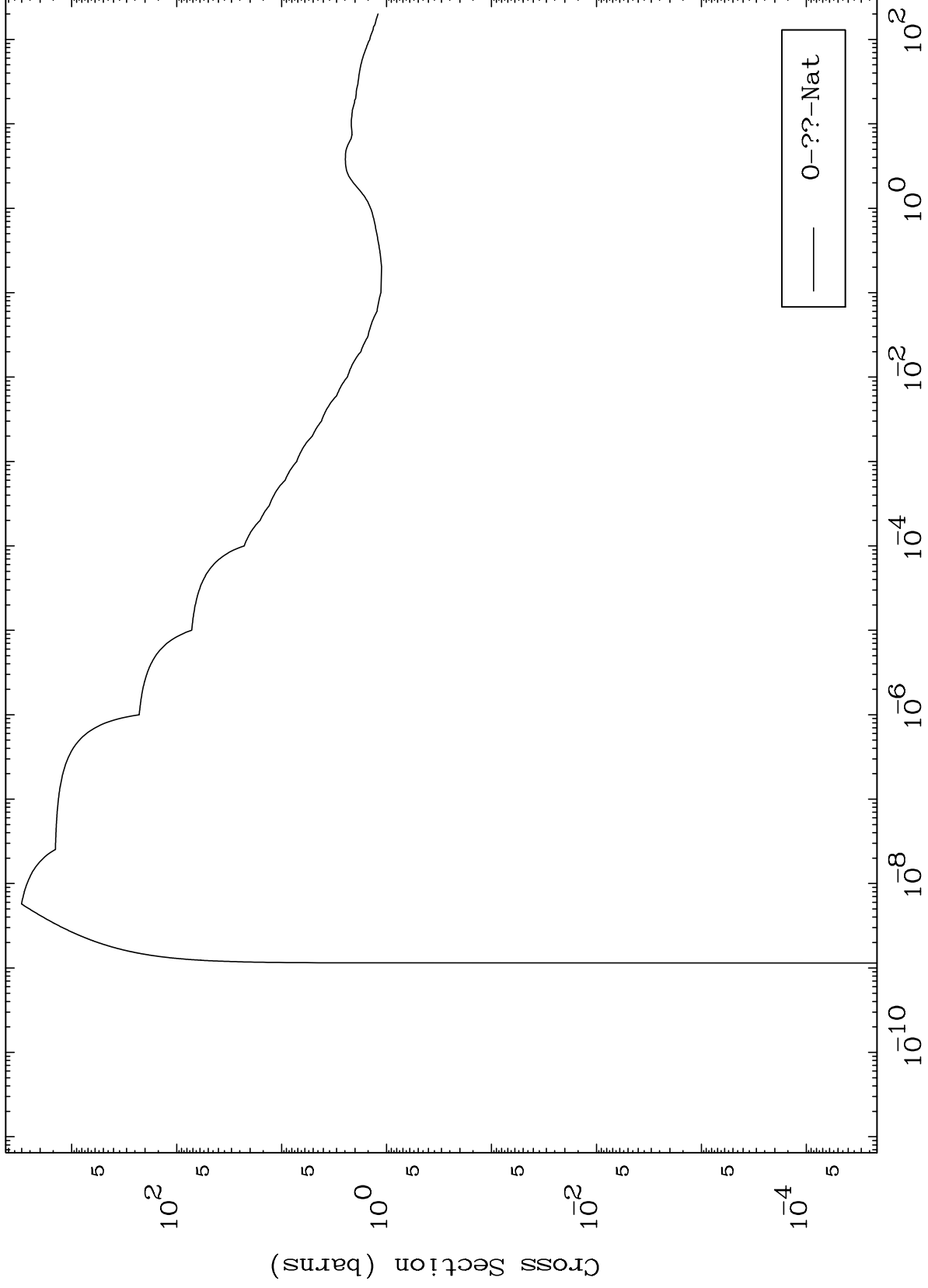


Incident Energy (MeV)

85-At-198

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Fission
Radionuclide Production Cross Section



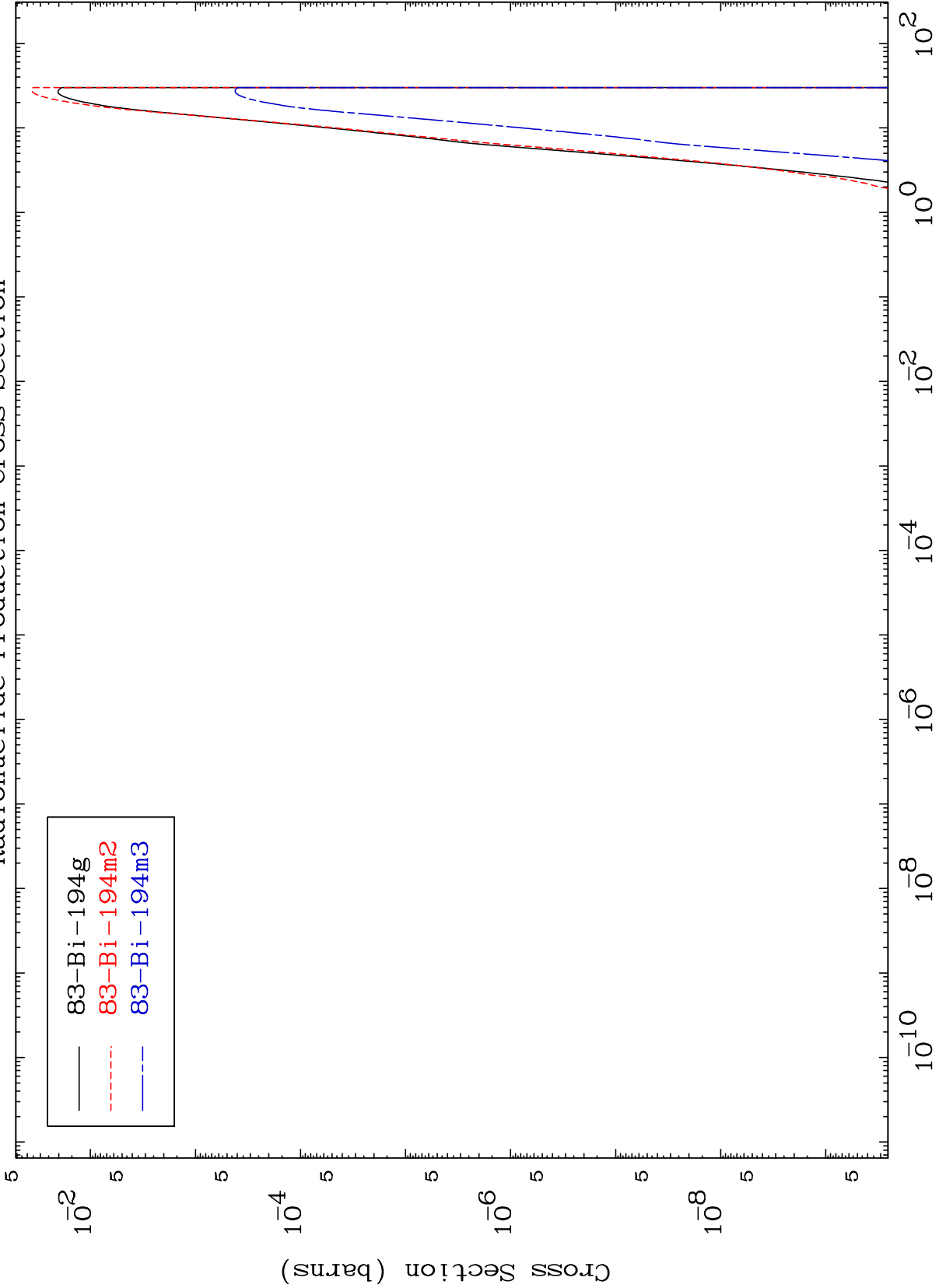
— 0-??-Nat

MAT 8511

$(n, n') \alpha$

85-At-198

Radionuclide Production Cross Section

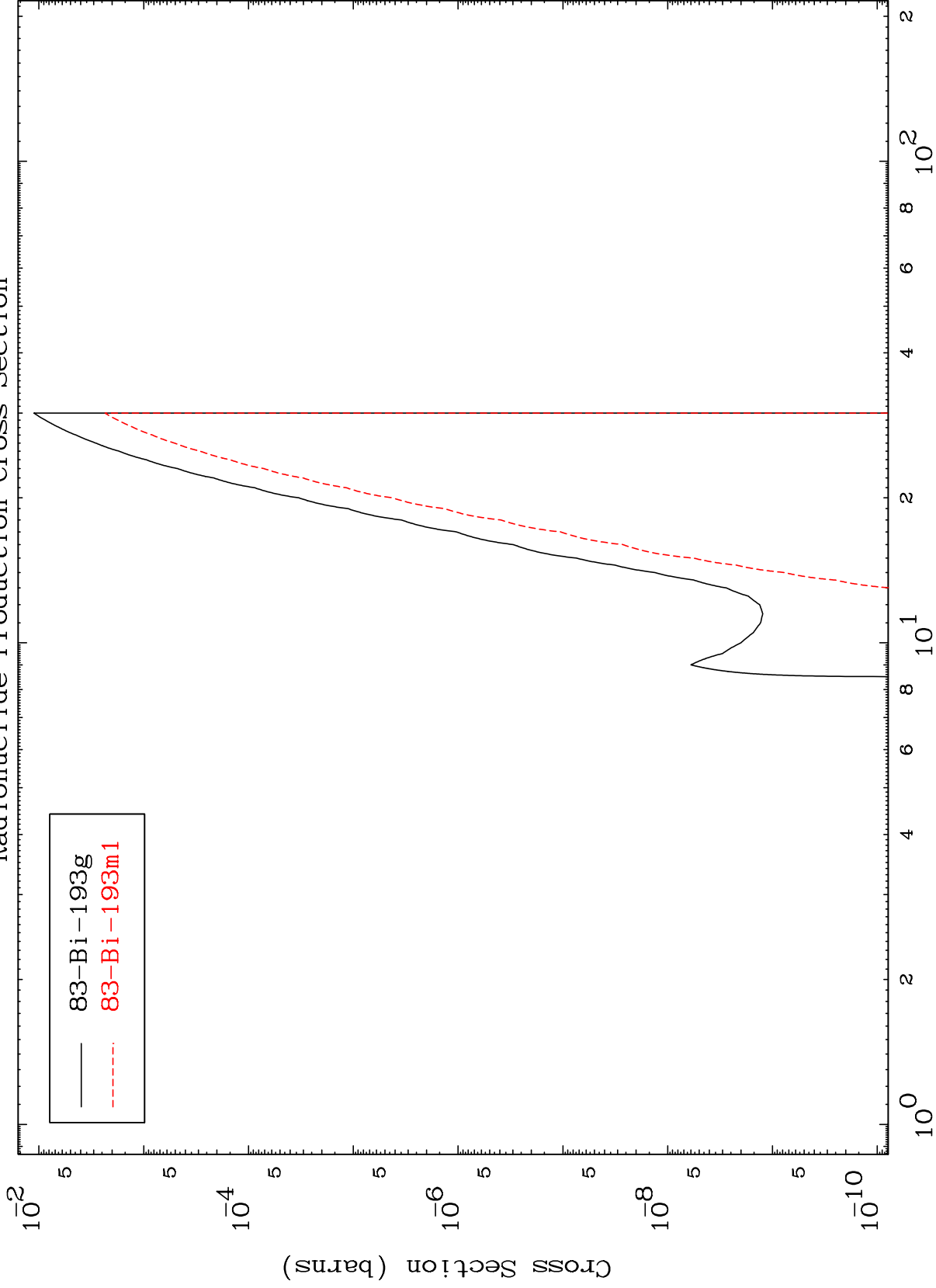


MAT 8511

(n,2n) α

85-At-198

Radionuclide Production Cross Section



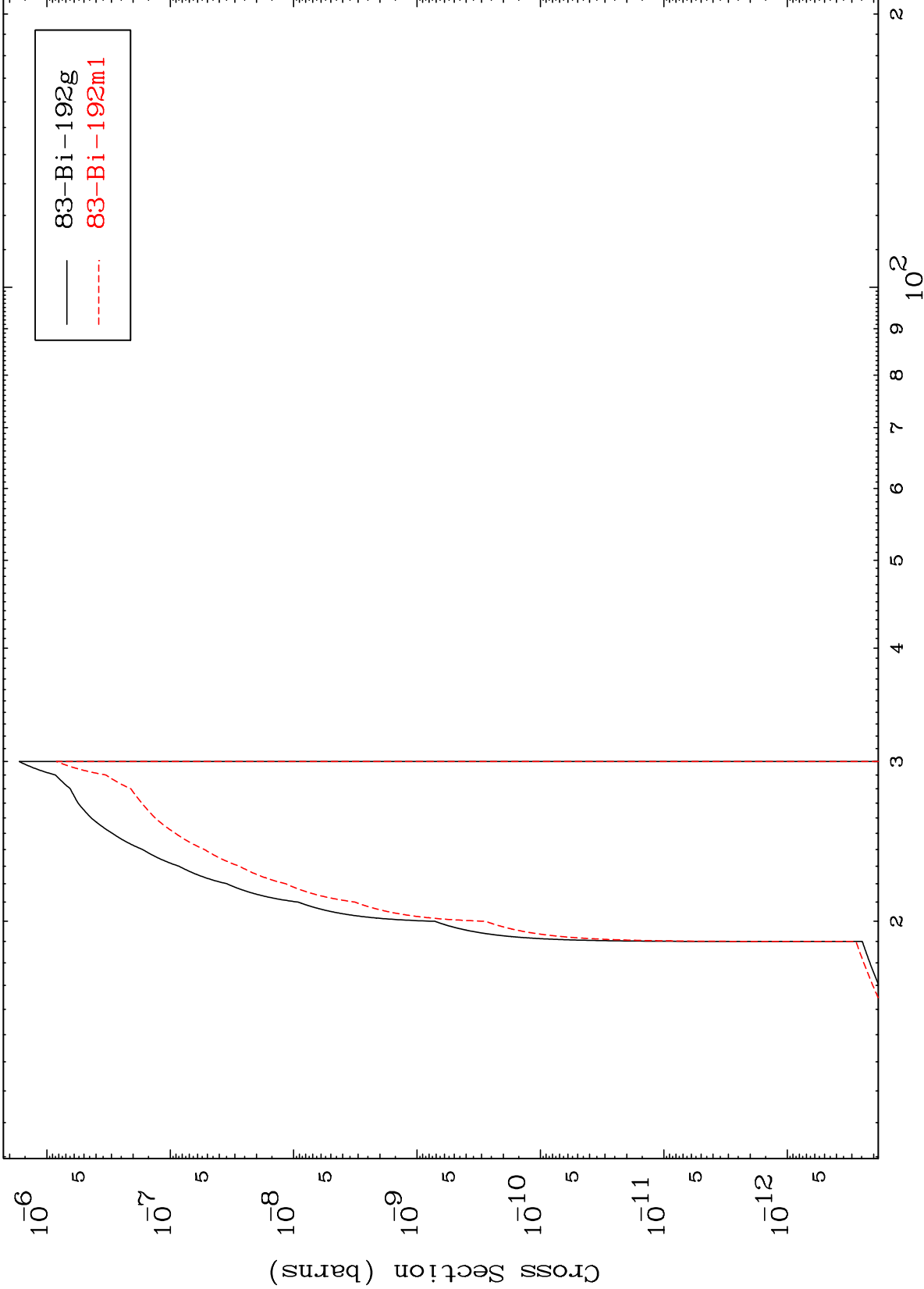
83-Bi-193g
83-Bi-193m1

Incident Energy (MeV)

85-At-198

22

Radionuclide Production Cross Section

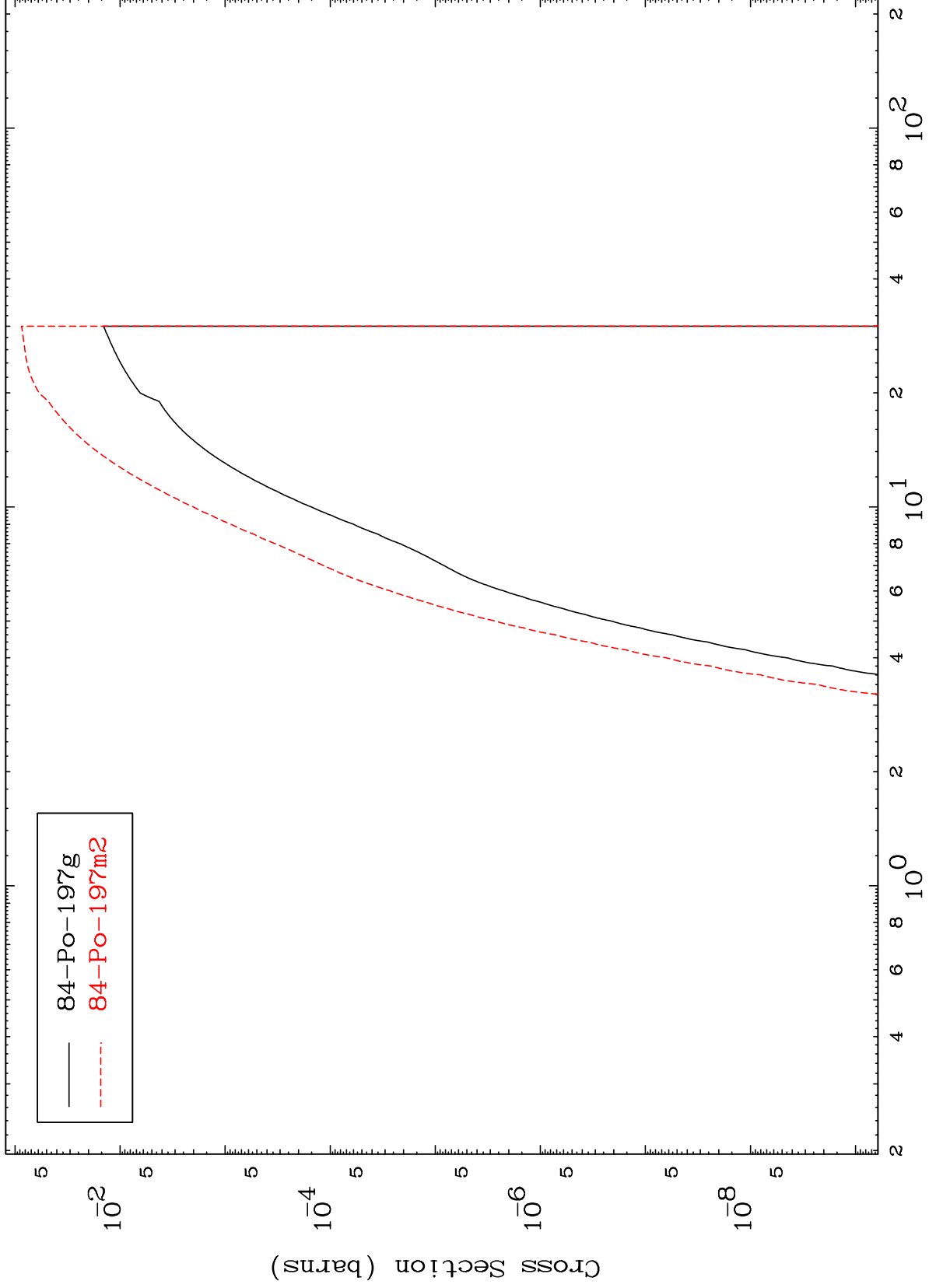


MAT 8511

(n,n') p

85-At-198

Radionuclide Production Cross Section



24

Incident Energy (MeV)

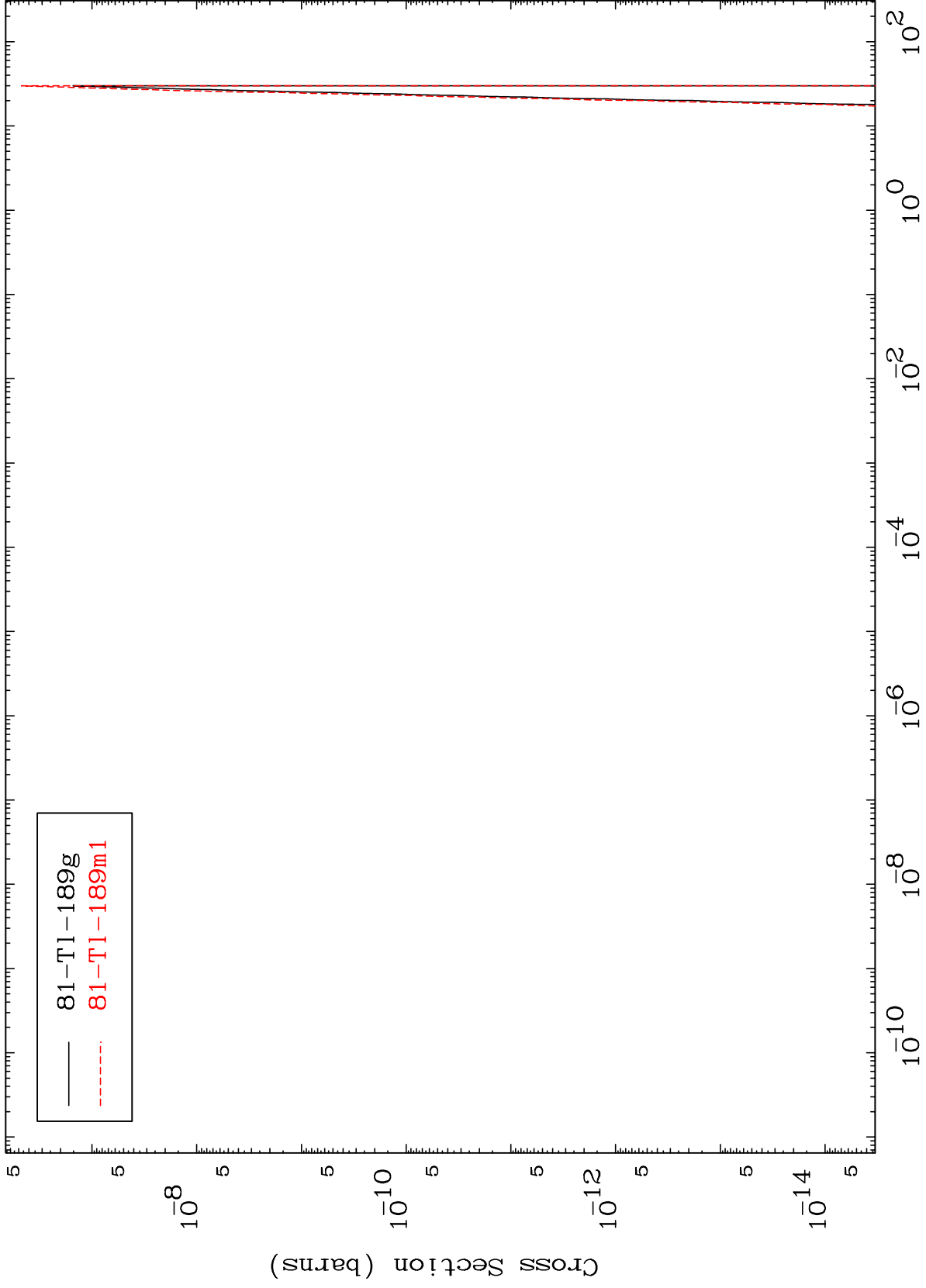
85-At-198

MAT 8511

(n,2n) 2α

85-At-198

Radionuclide Production Cross Section



25

Incident Energy (MeV)

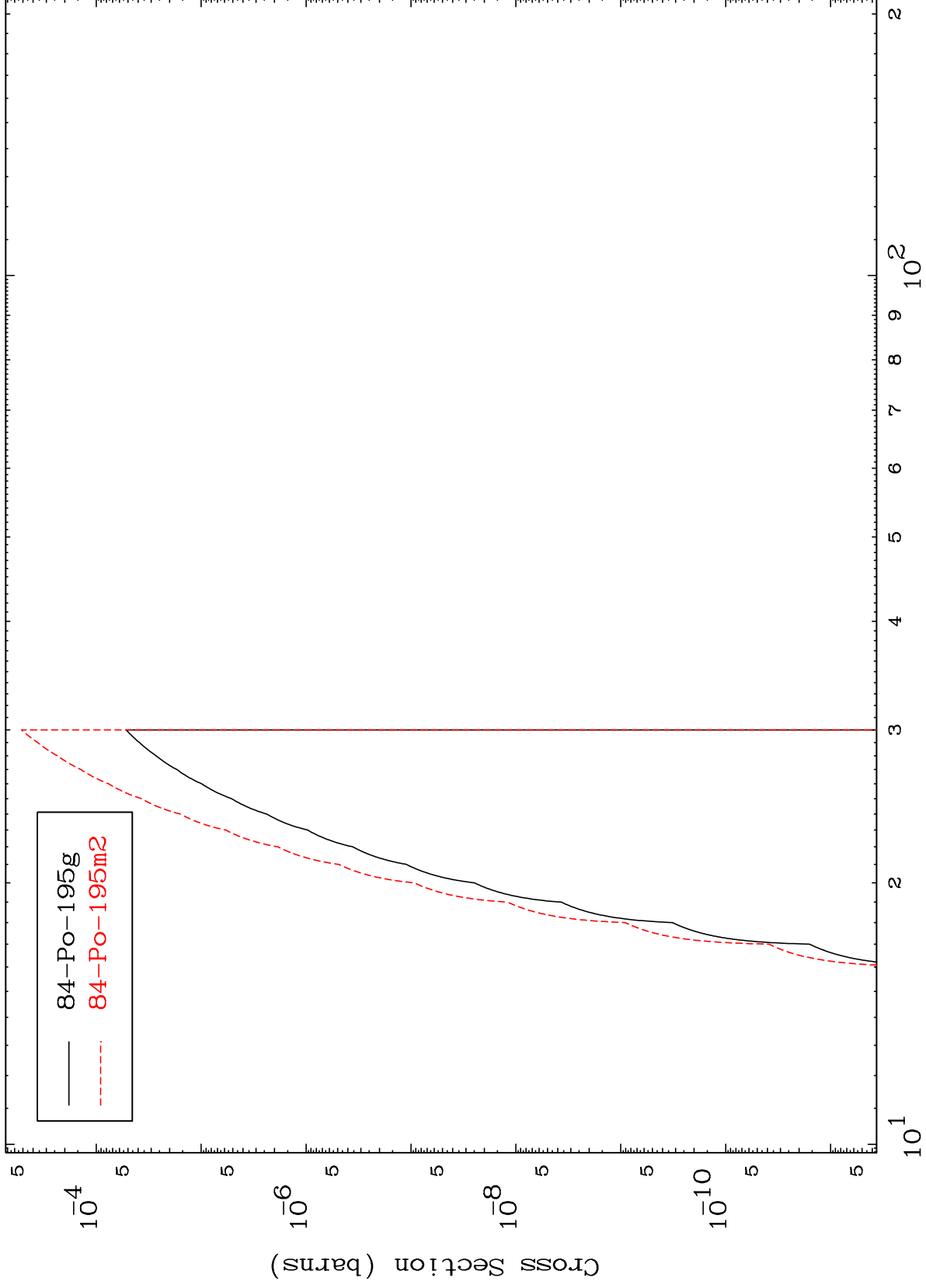
85-At-198

MAT 8511

(n,n') t

85-At-198

Radionuclide Production Cross Section



26

Incident Energy (MeV)

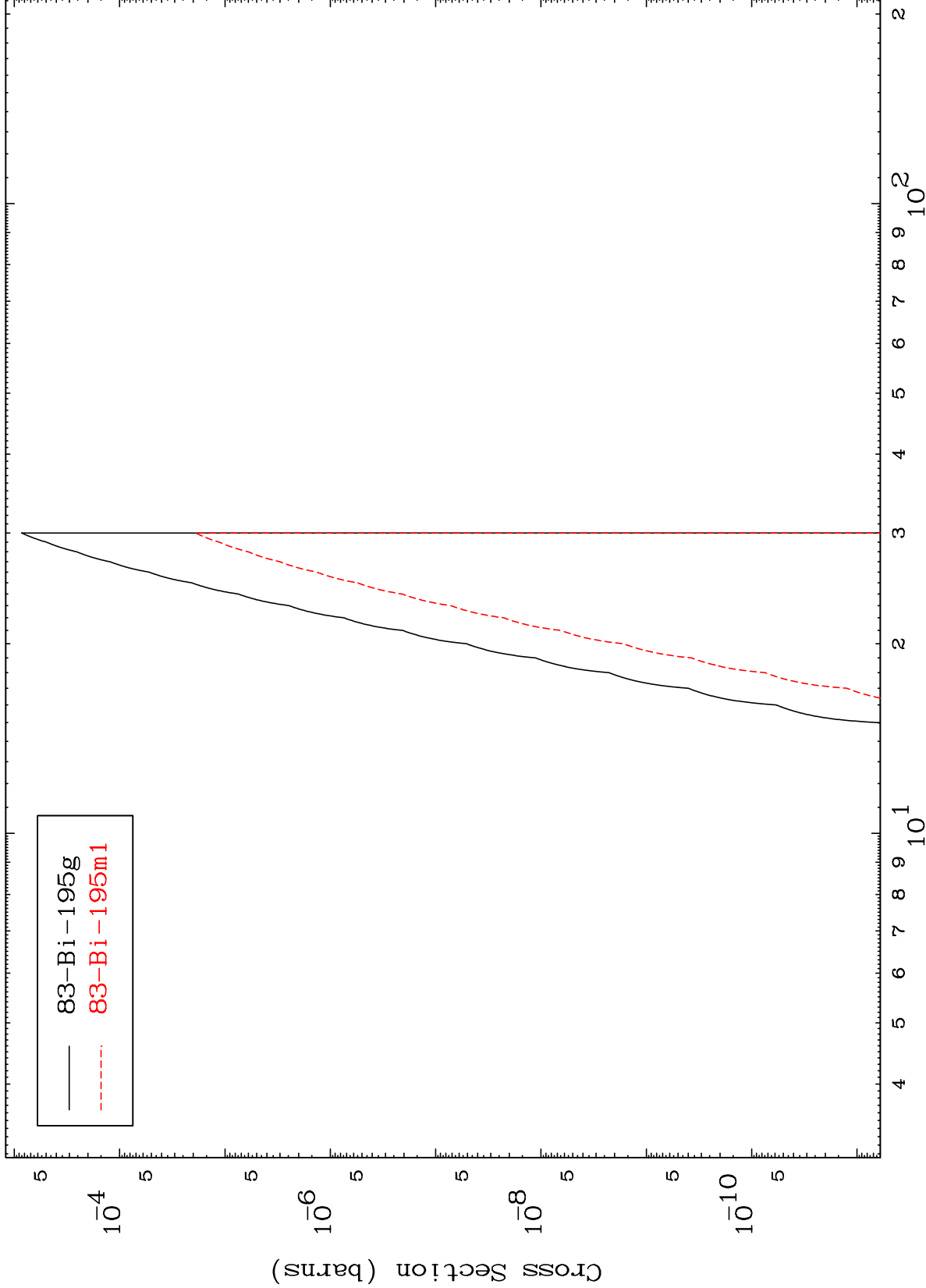
85-At-198

MAT 8511

(n,n') He-3

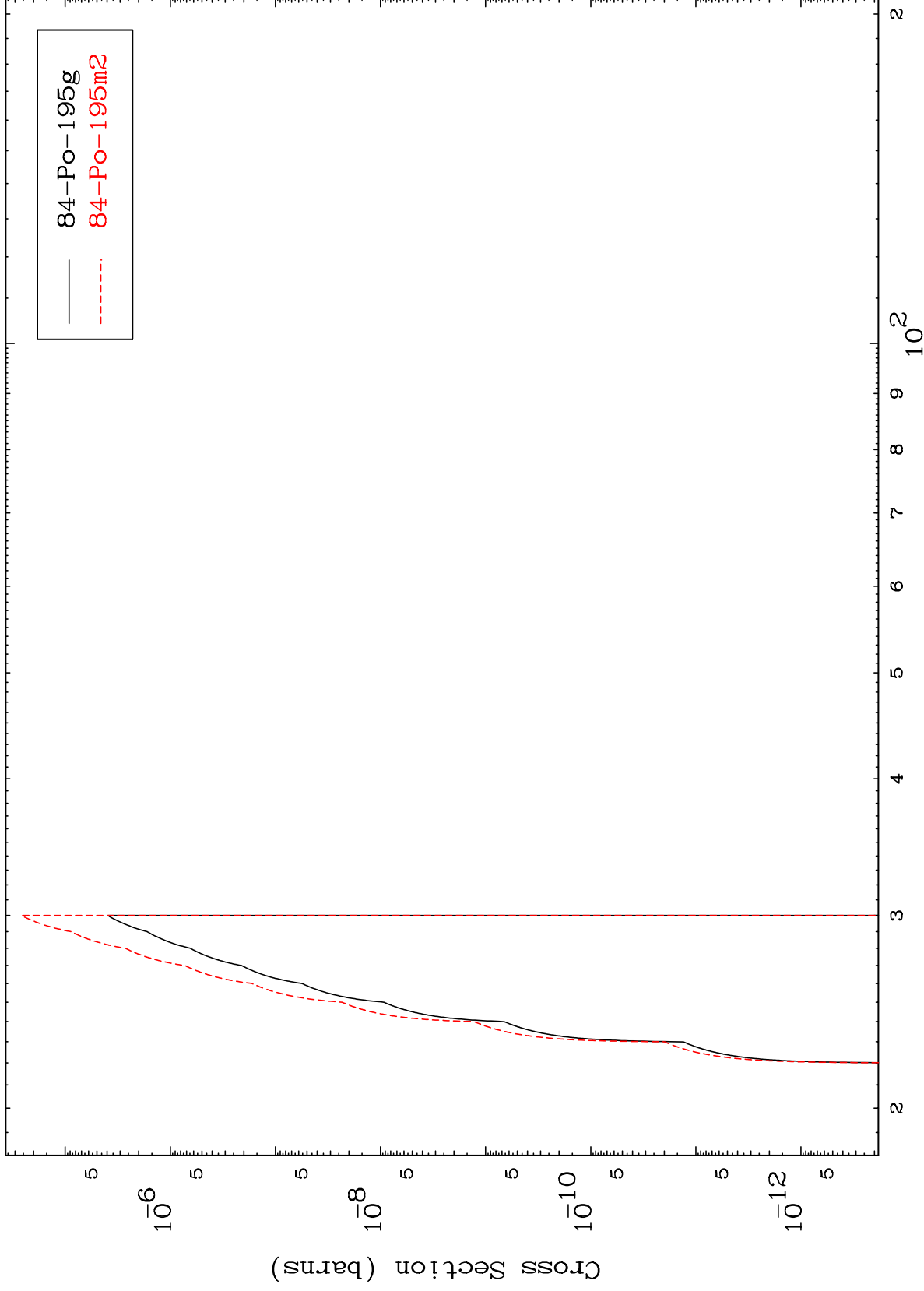
85-At-198

Radionuclide Production Cross Section

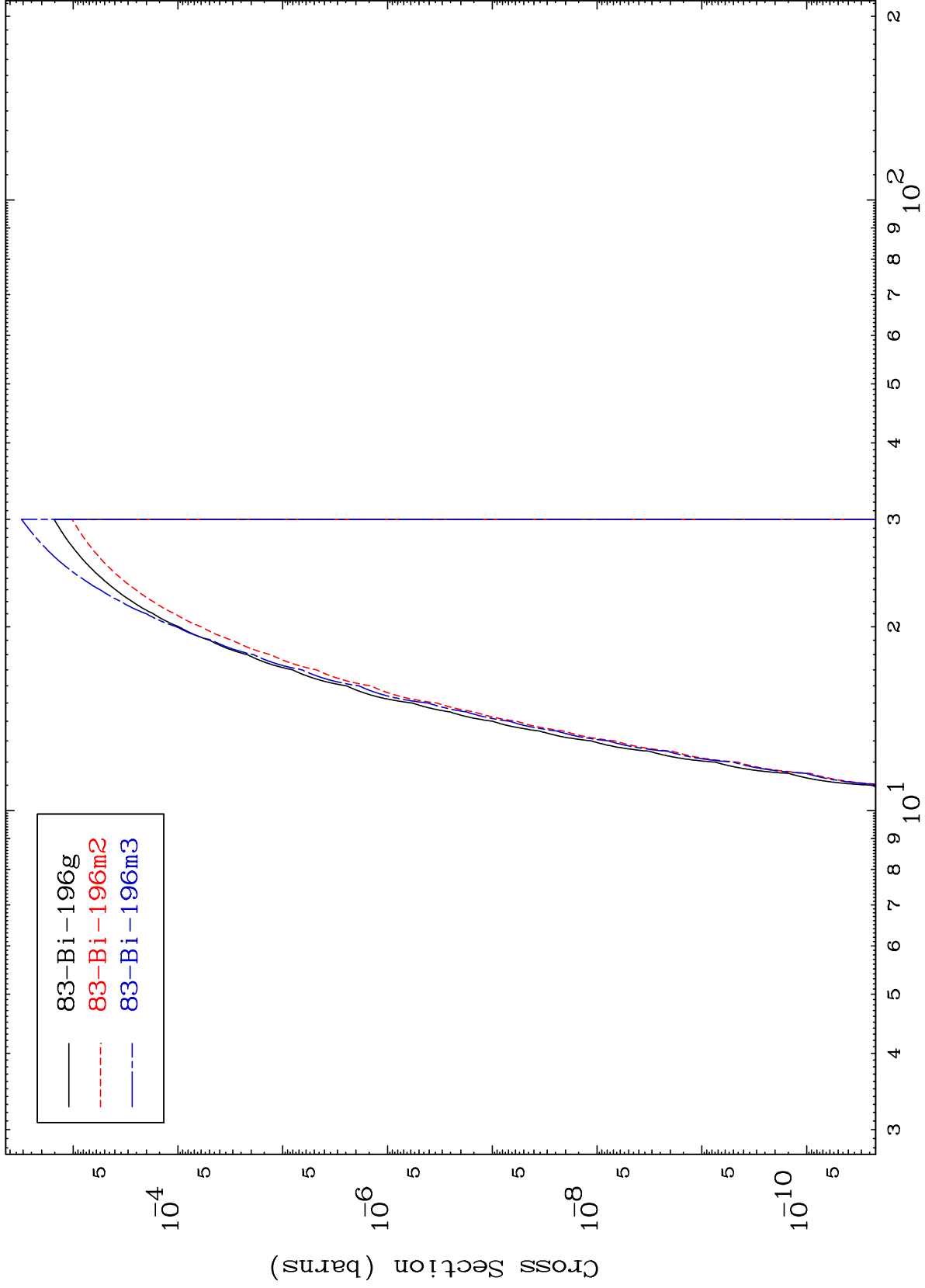


— 83-Bi-195g
- - - 83-Bi-195m1

Radionuclide Production Cross Section



Radionuclide Production Cross Section

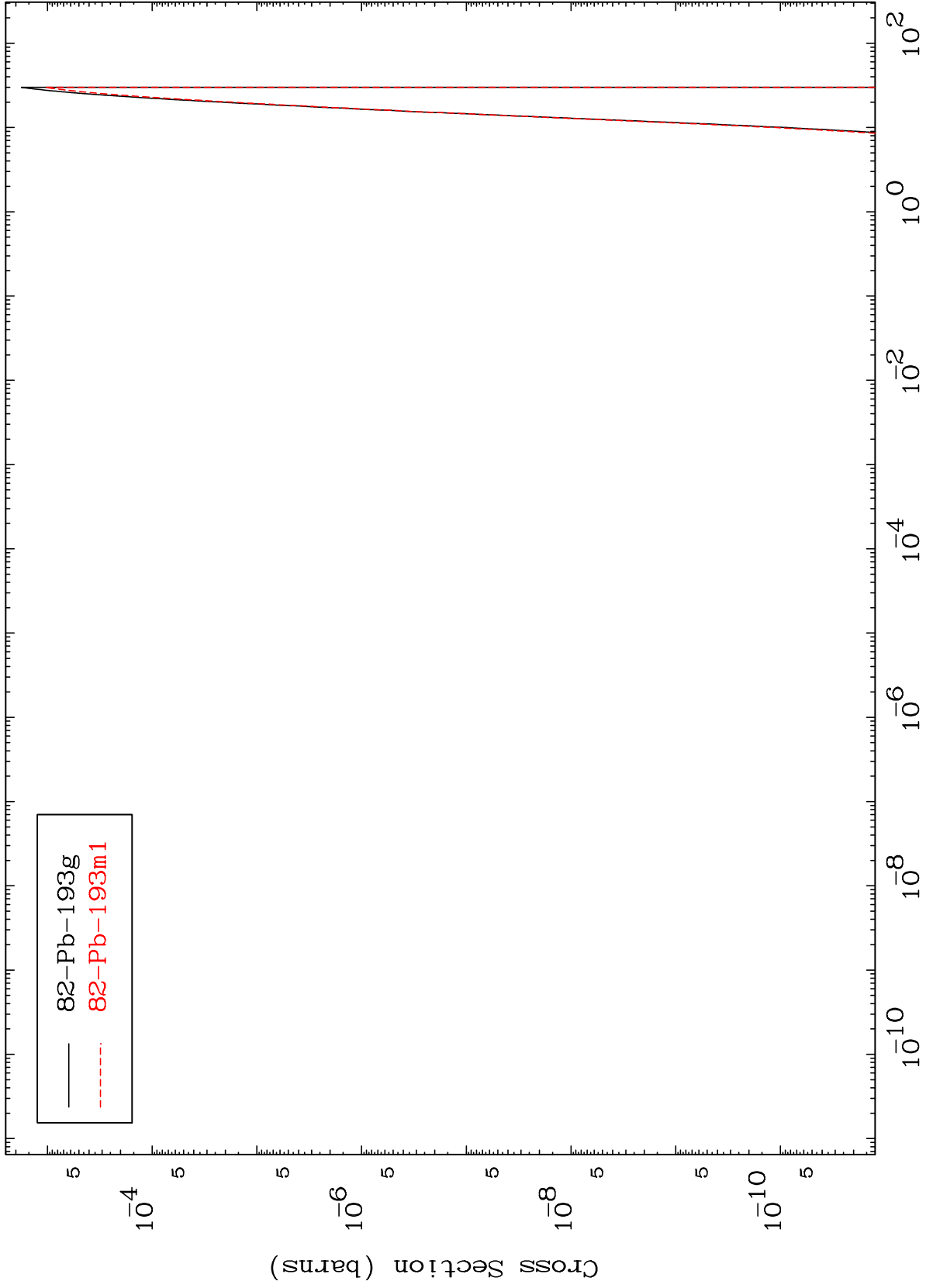


MAT 8511

(n,n') p α

85-At-198

Radionuclide Production Cross Section



30

Incident Energy (MeV)

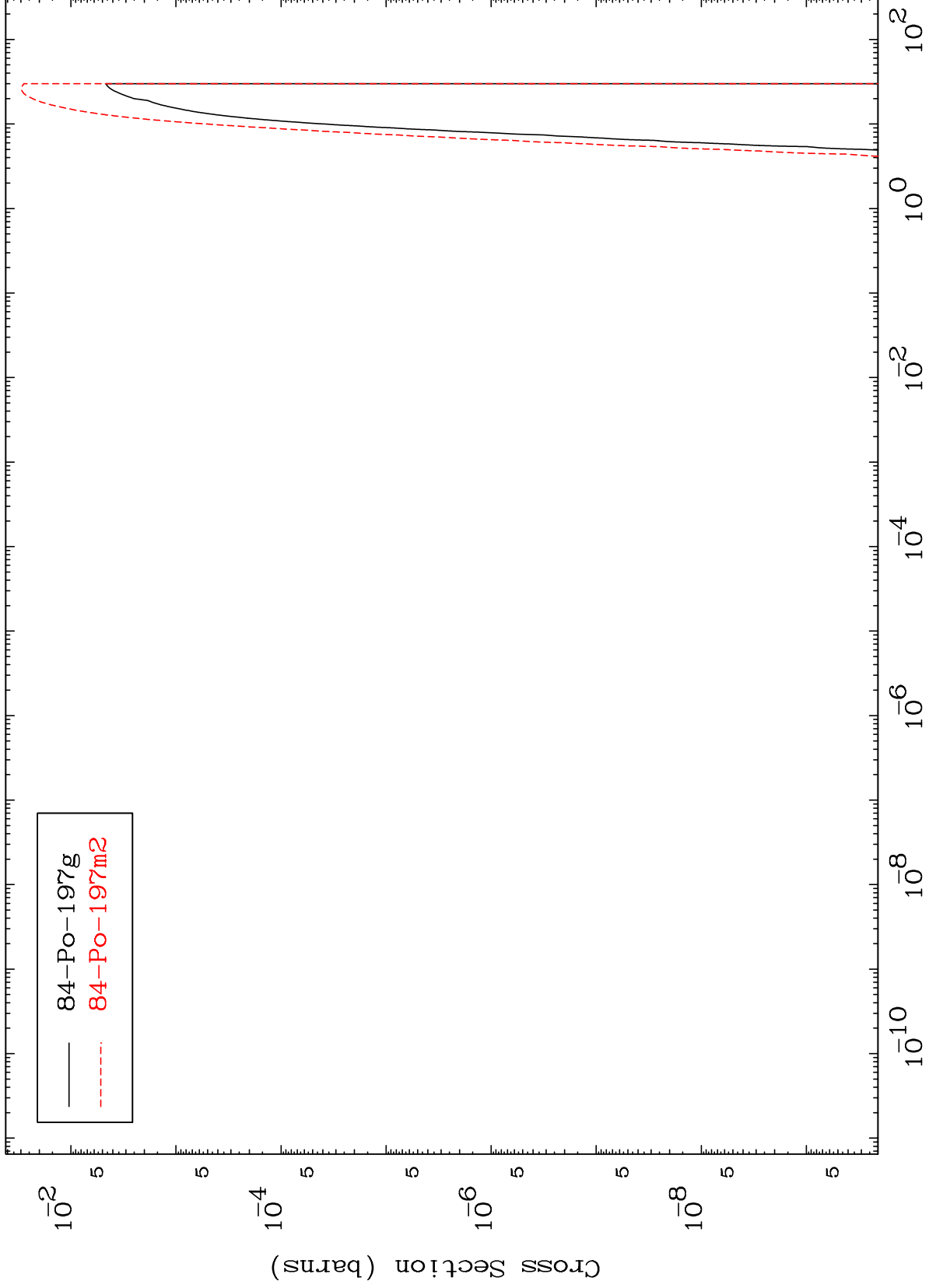
85-At-198

MAT 8511

(n,d)

85-At-198

Radionuclide Production Cross Section

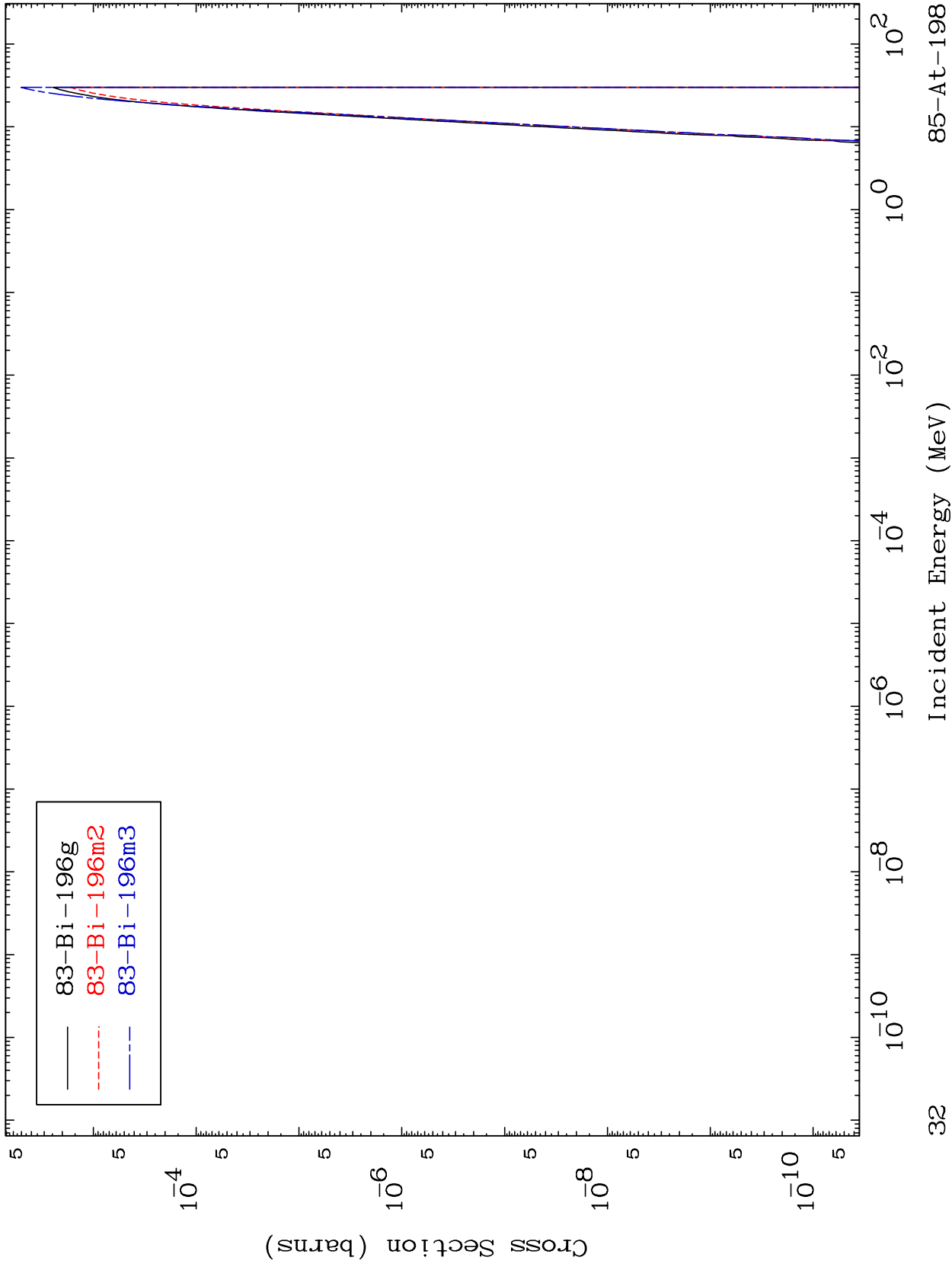


MAT 8511

(n,He-3)

85-At-198

Radionuclide Production Cross Section

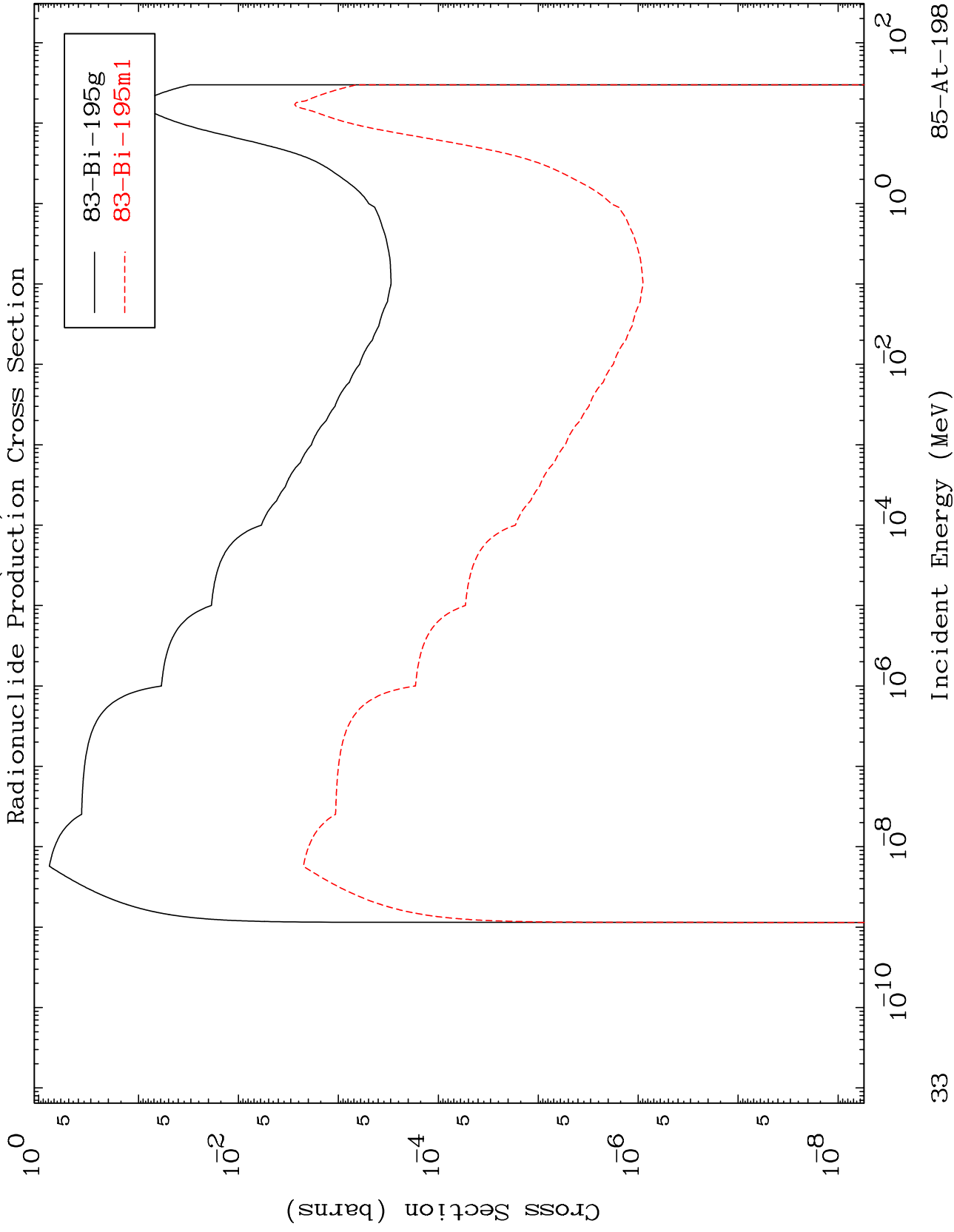


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Incident Energy (MeV)

85-At-198

(n, α)
Radionuclide Production Cross Section

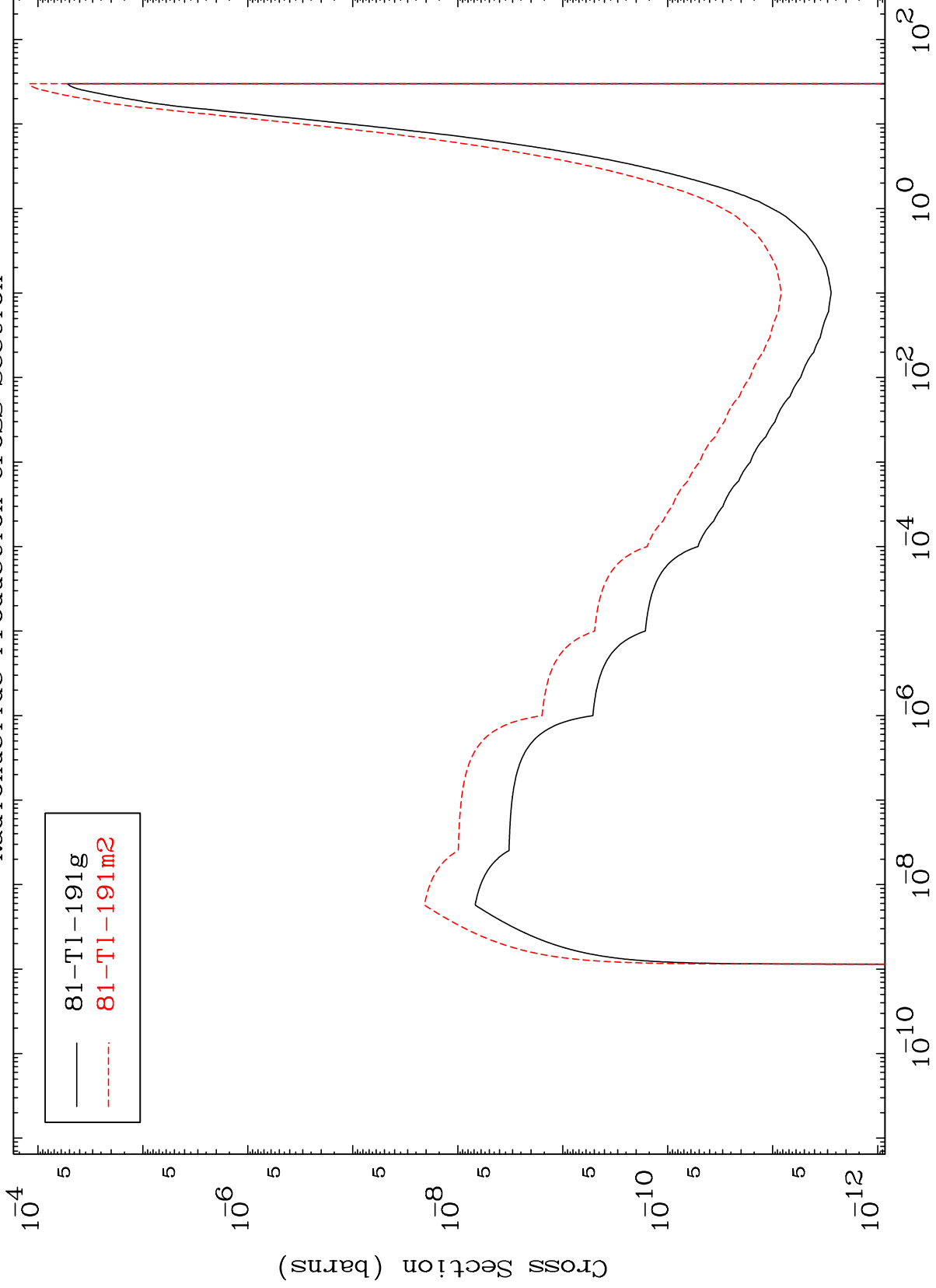


MAT 85111

(n,2α)

85-At-198

Radionuclide Production Cross Section



34

Incident Energy (MeV)

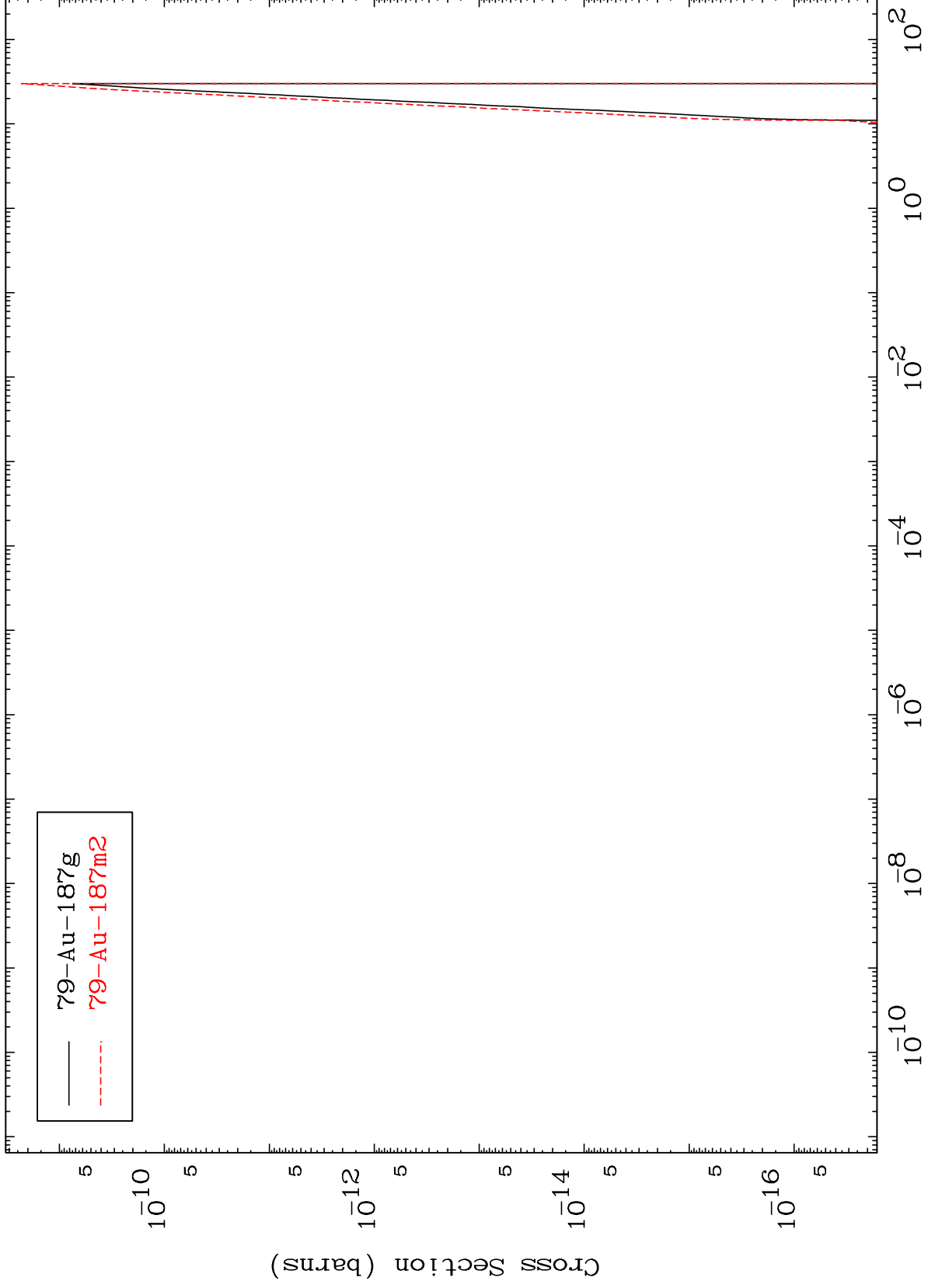
85-At-198

MAT 8511

(n,3 α)

85-At-198

Radionuclide Production Cross Section



35

Incident Energy (MeV)

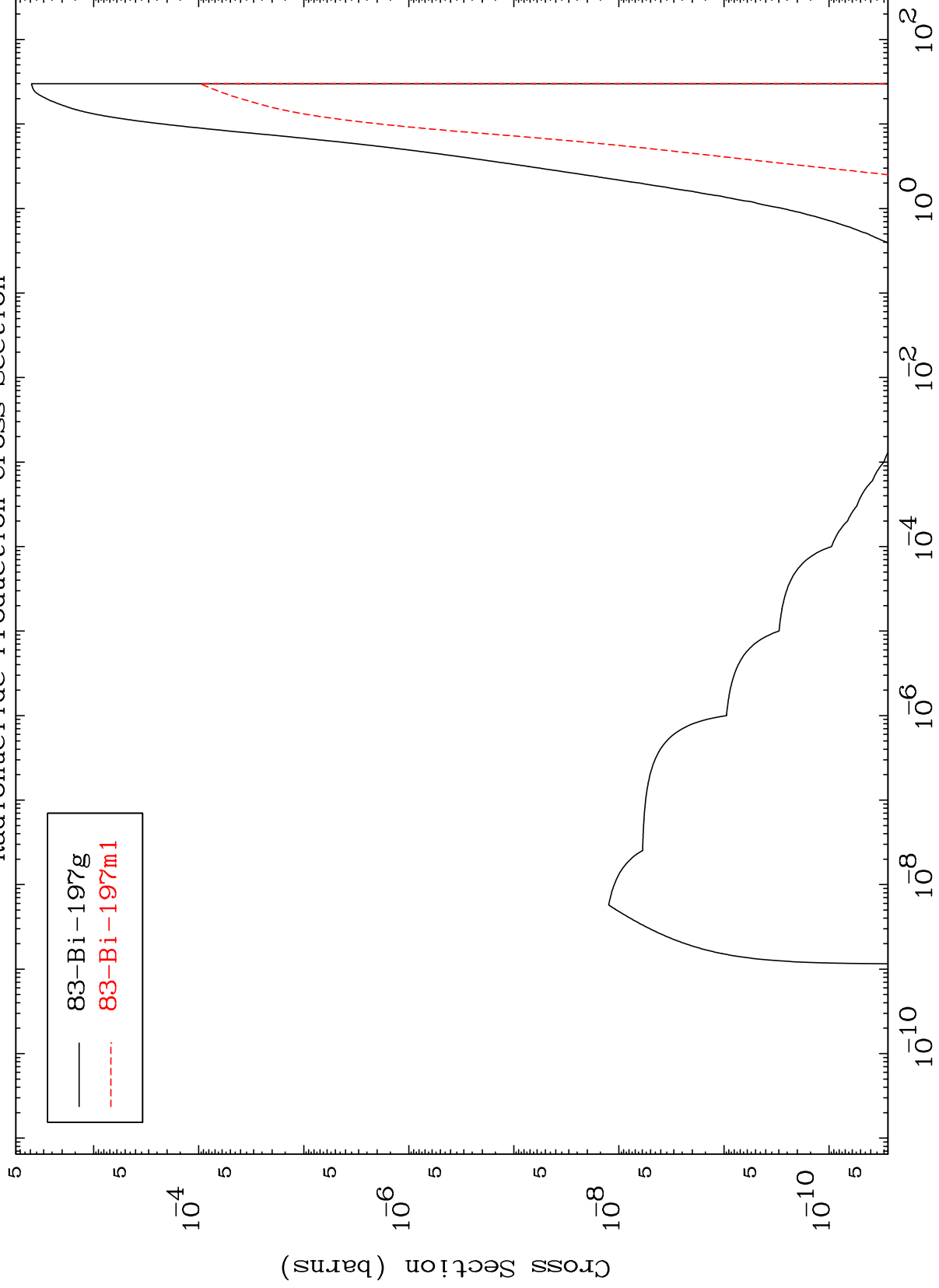
85-At-198

MAT 8511

(n,2p)

85-At-198

Radionuclide Production Cross Section



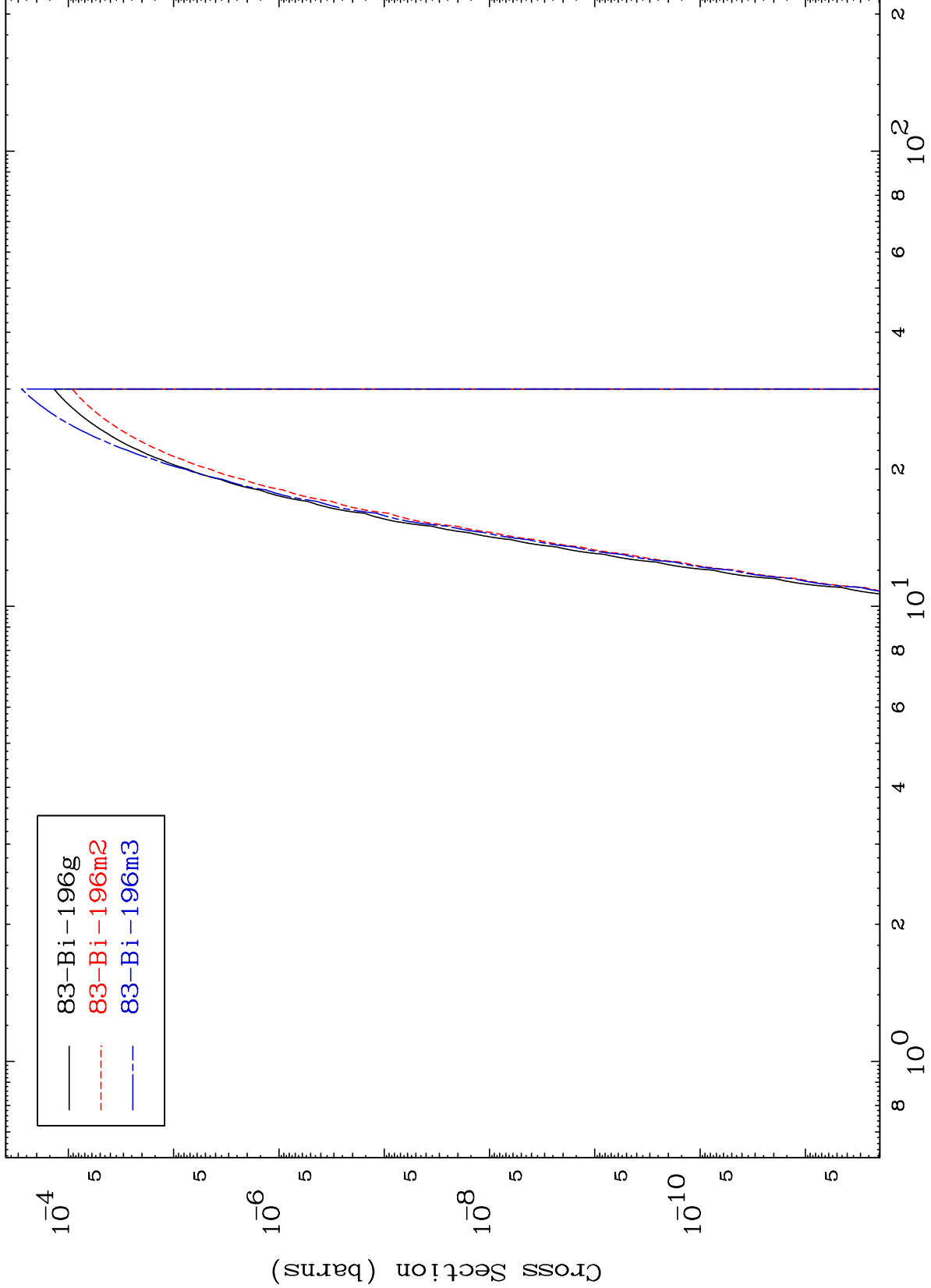
83-Bi-197g
83-Bi-197m1

Incident Energy (MeV)

85-At-198

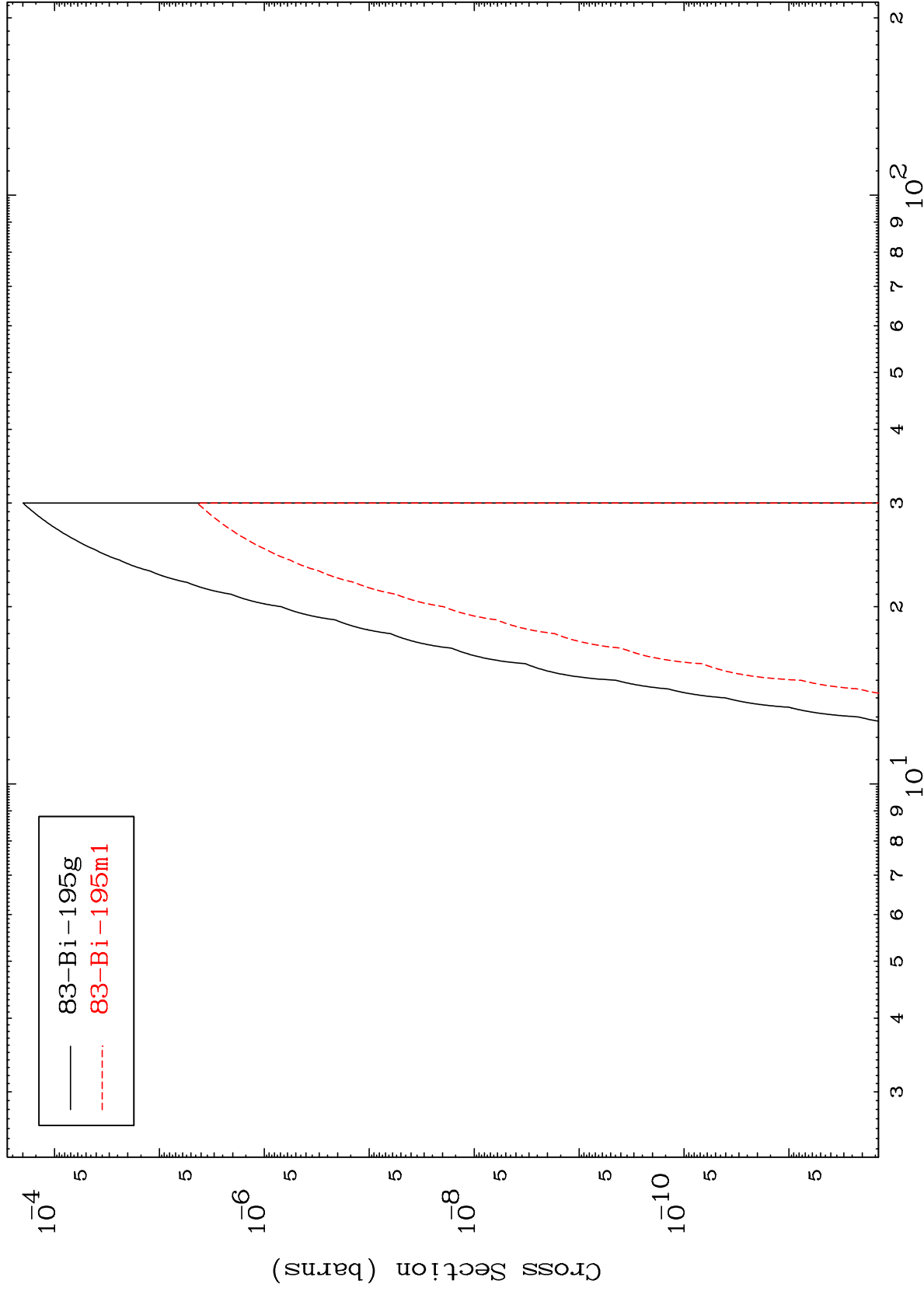
36

Radionuclide Production Cross Section



83-Bi-196g
83-Bi-196m2
83-Bi-196m3

Radionuclide Production Cross Section

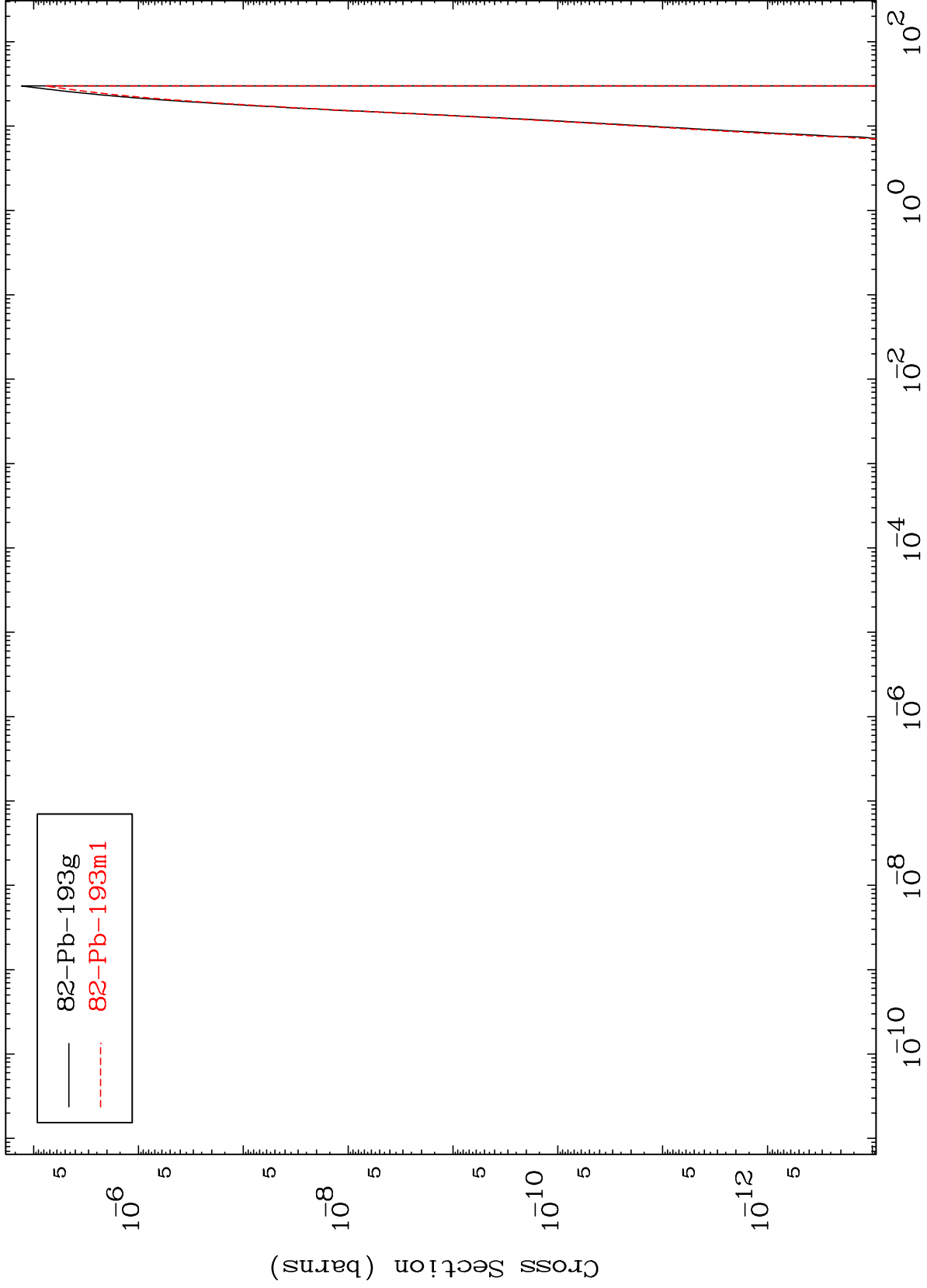


MAT 8511

(n,d) α

85-At-198

Radionuclide Production Cross Section



39

Incident Energy (MeV)

85-At-198