

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

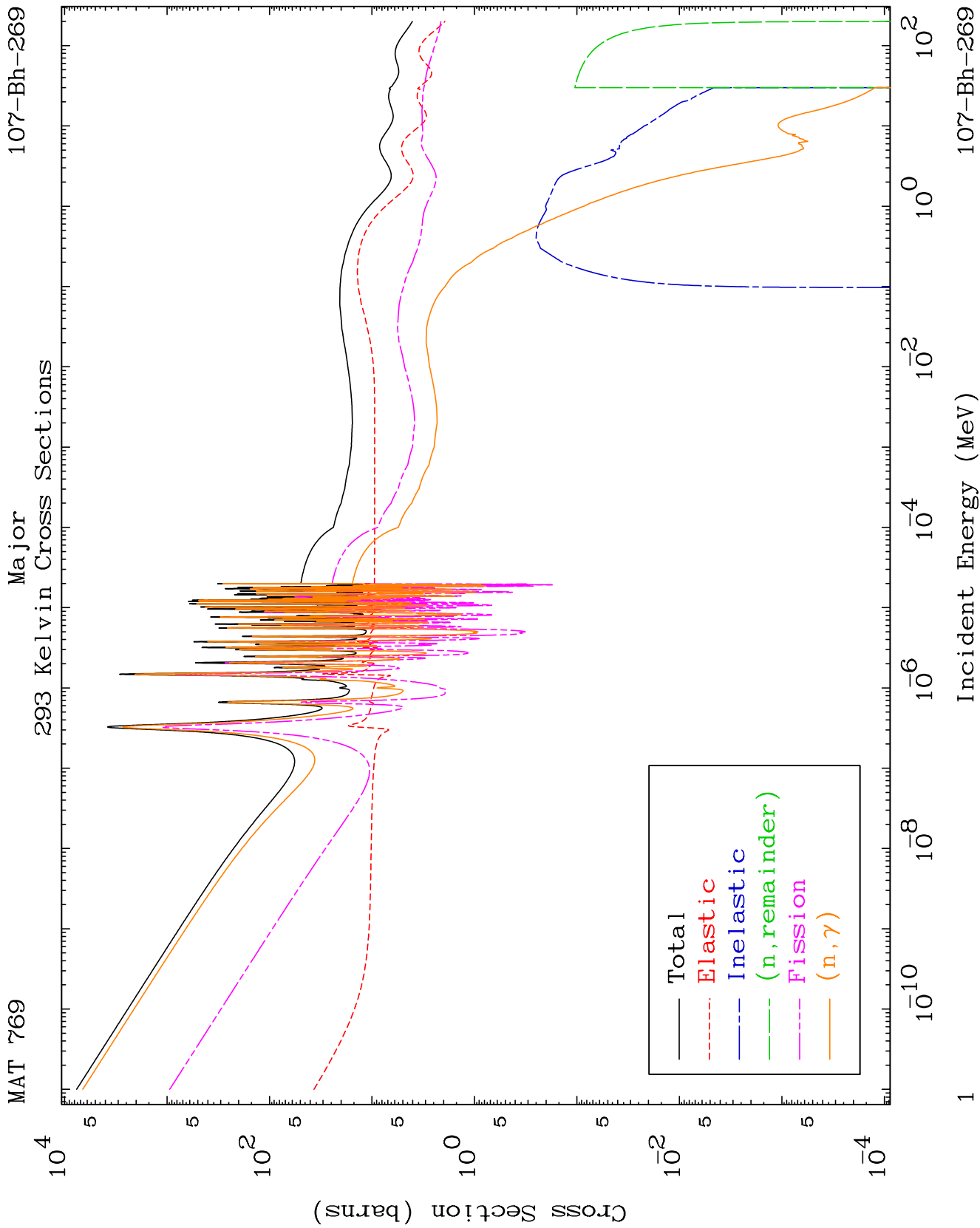
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Press Mouse Button to Start



MAT 769

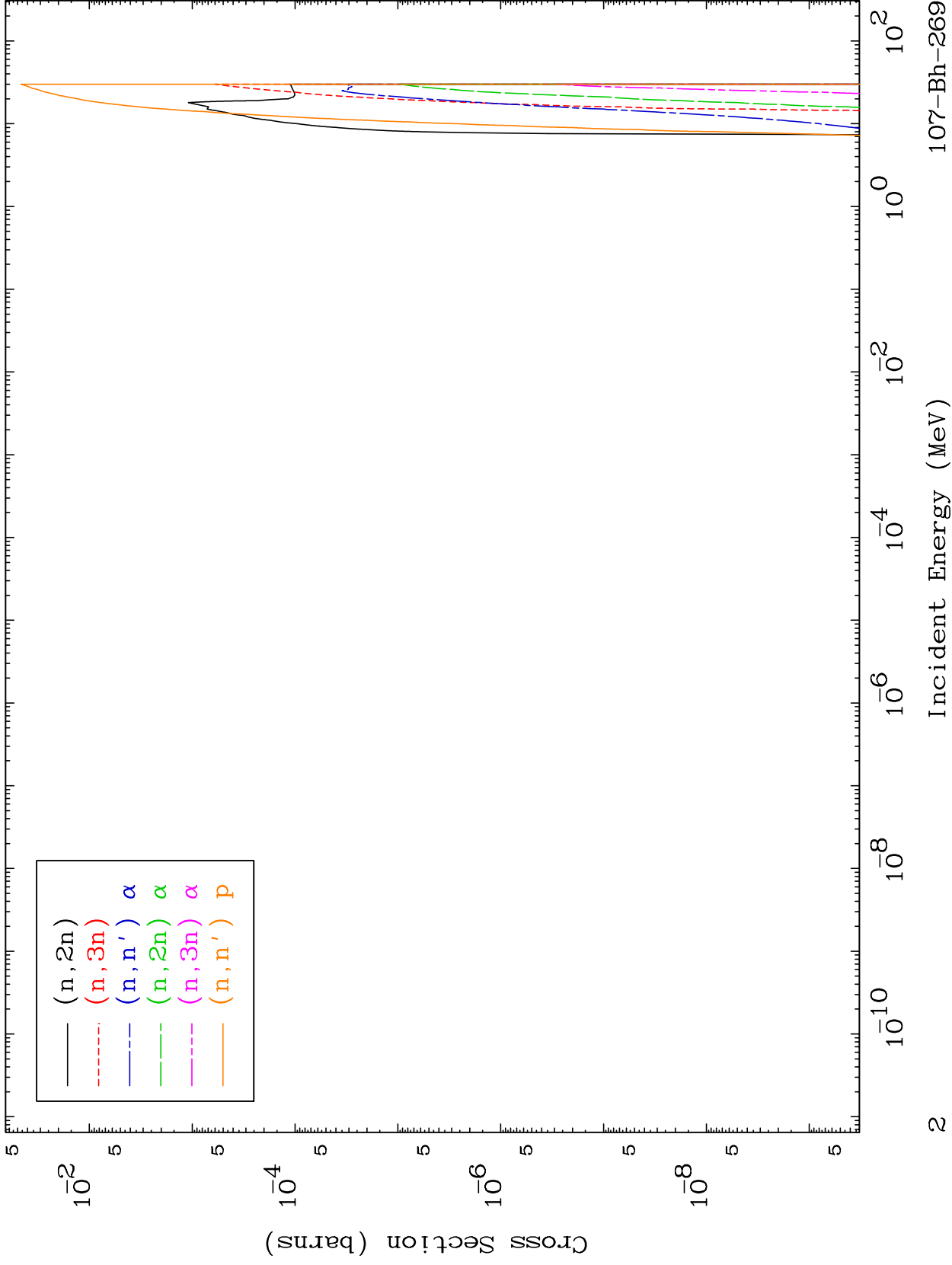
Major

107-Bh-269

MAT 769

Neutron Production
293 Kelvin Cross Sections

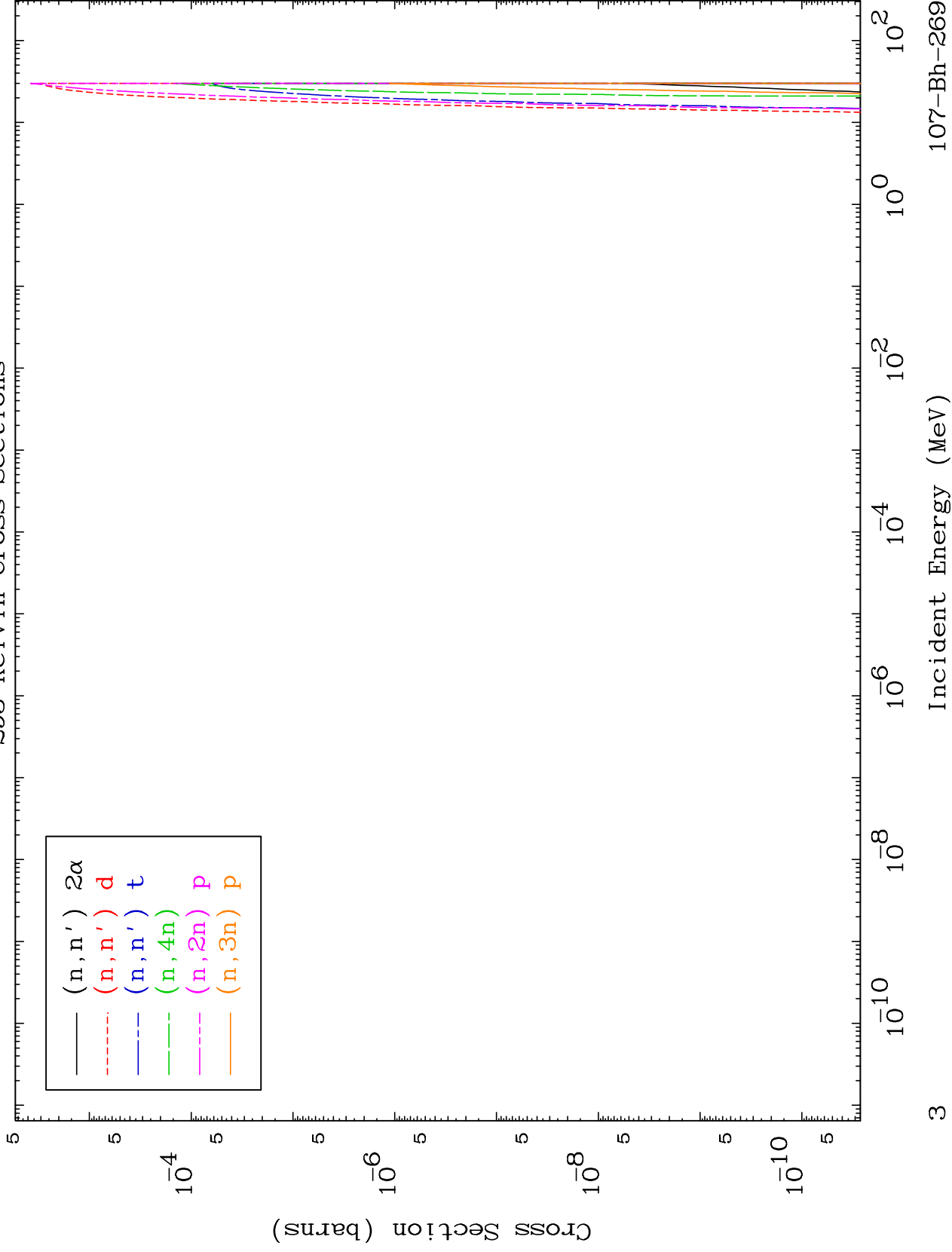
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MAT 769

Neutron Production
293 Kelvin Cross Sections

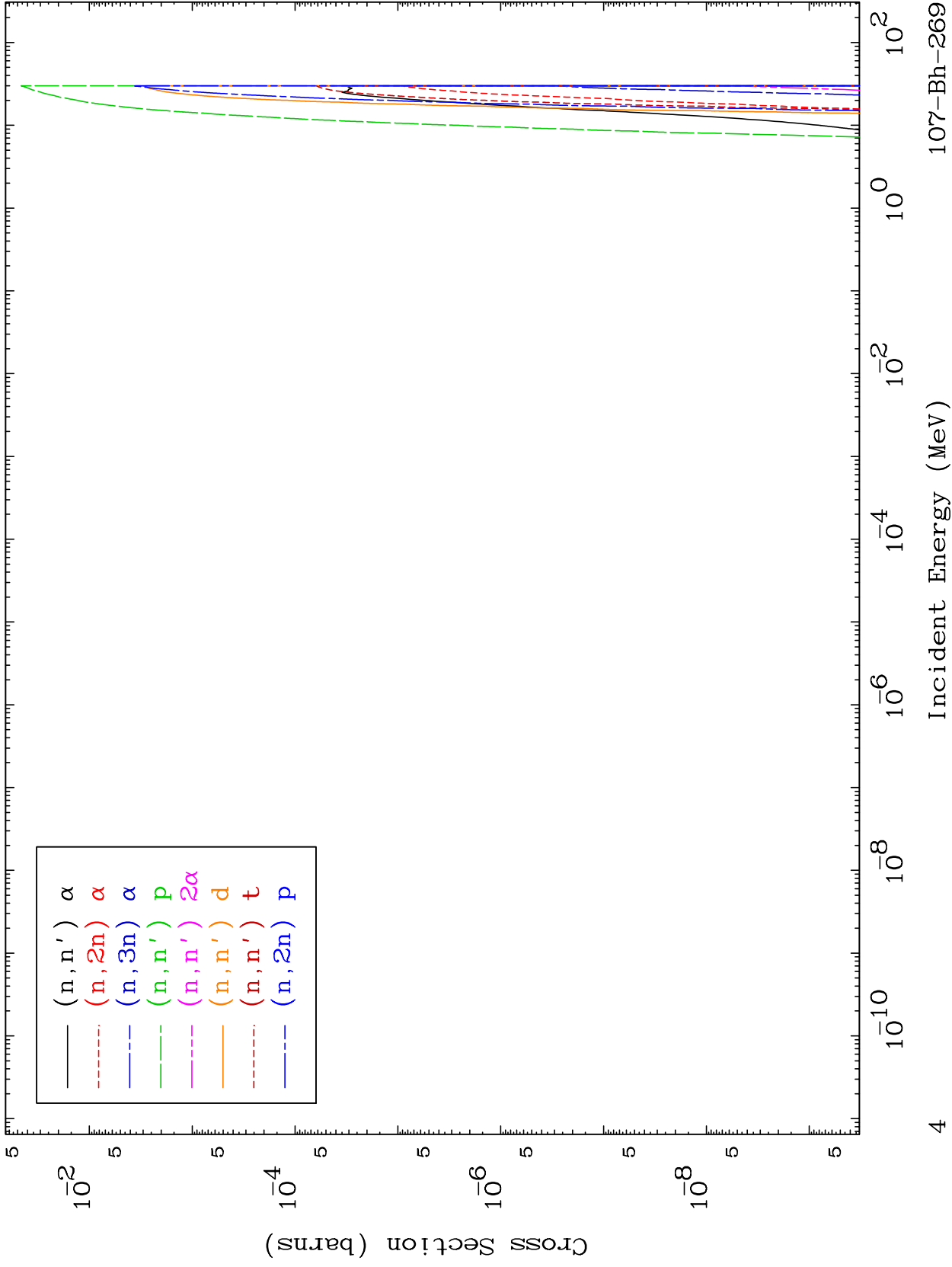
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Charged Particle
293 Kelvin Cross Sections

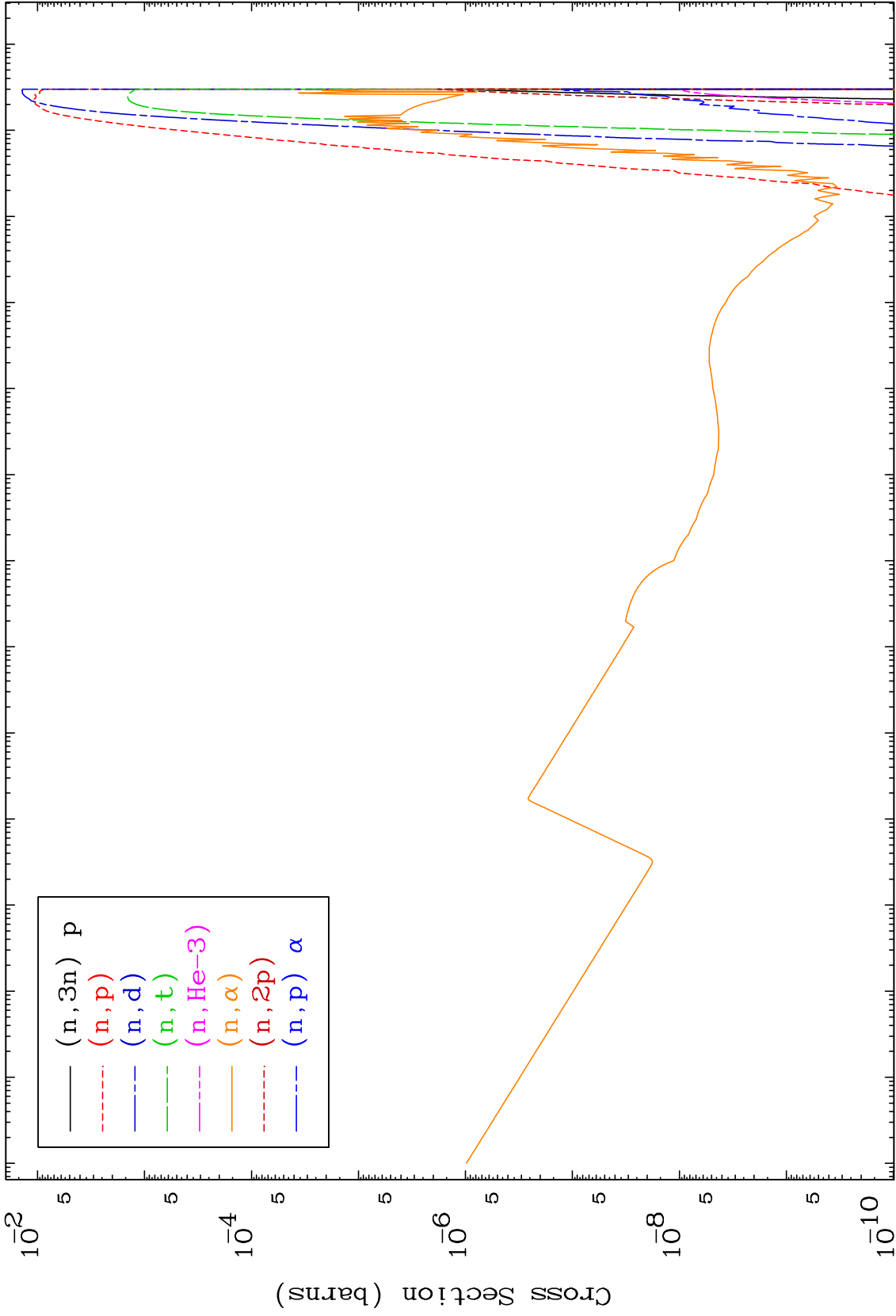
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Charged Particle
293 Kelvin Cross Sections

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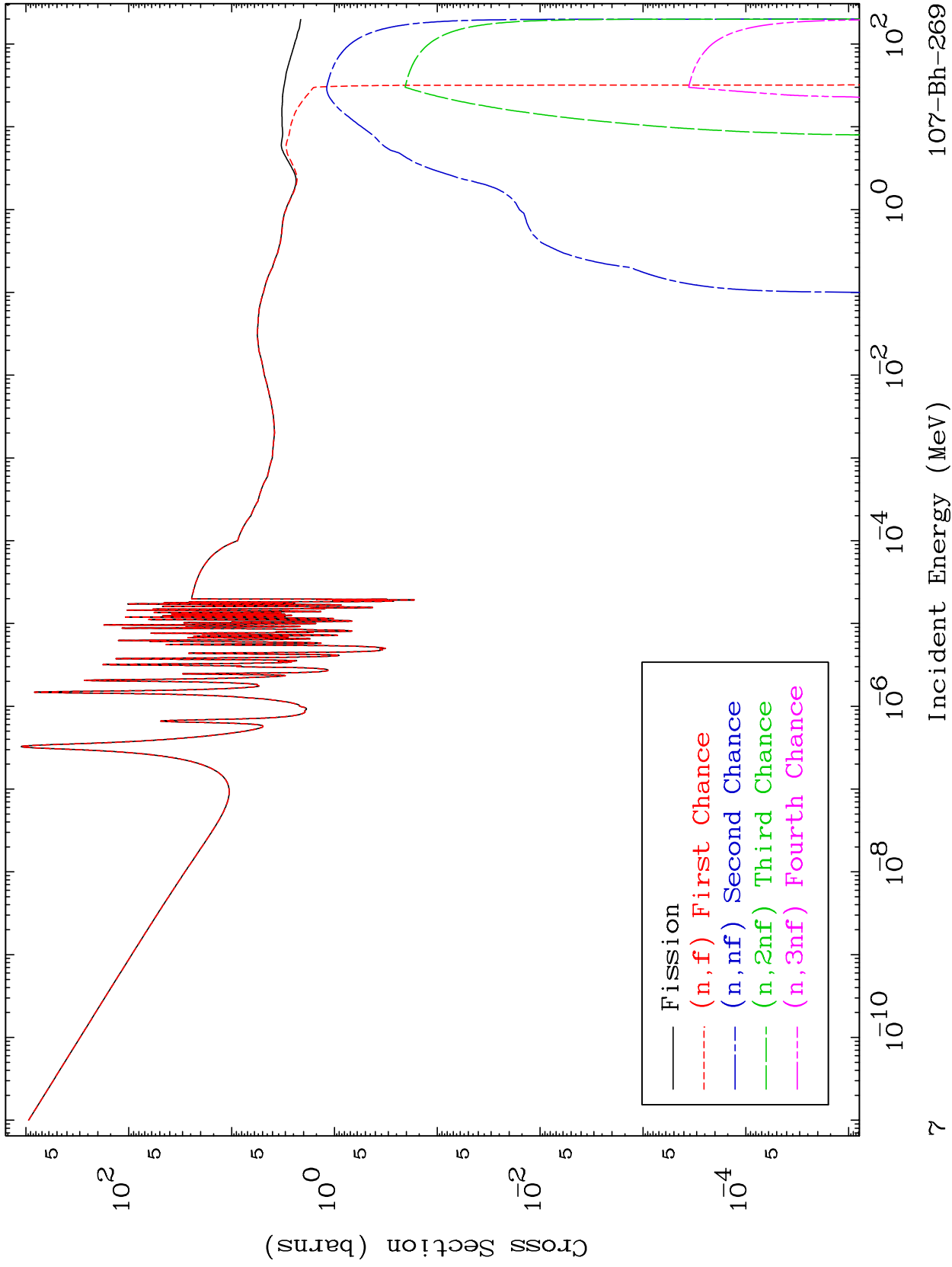
Incident Energy (MeV)

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Fission
293 Kelvin Cross Sections

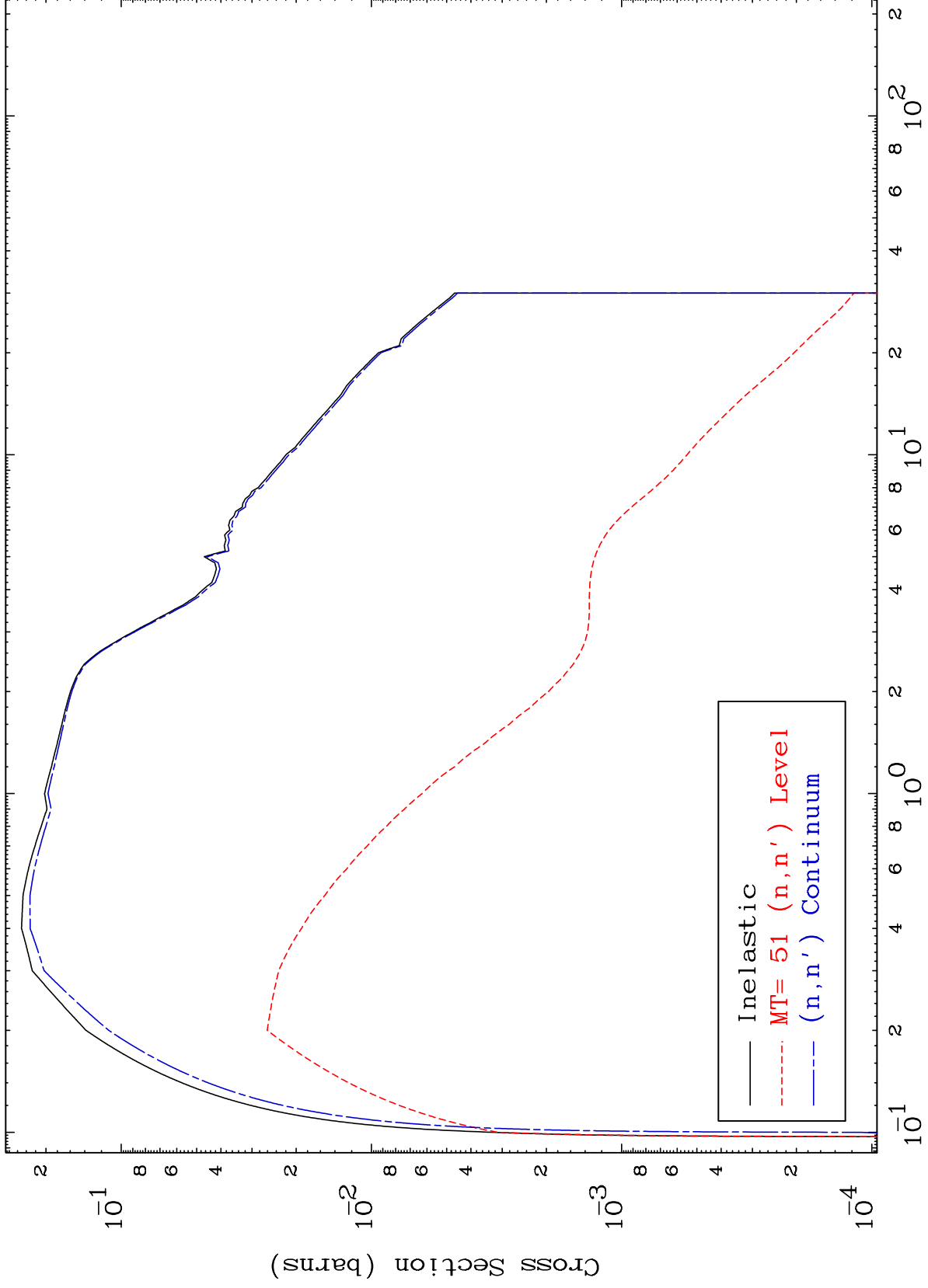
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(n,n') Level
293 Kelvin Cross Sections

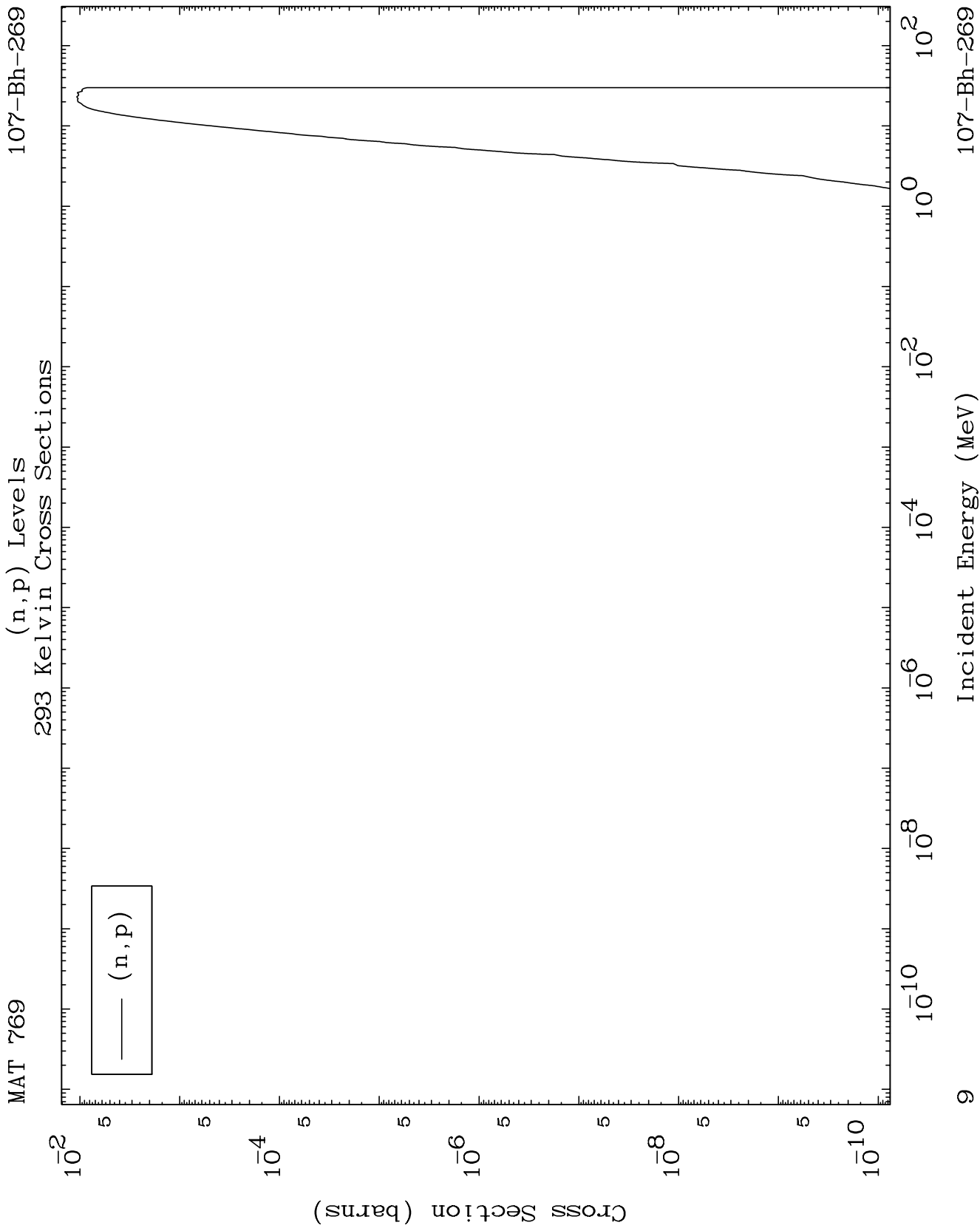
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Incident Energy (MeV)

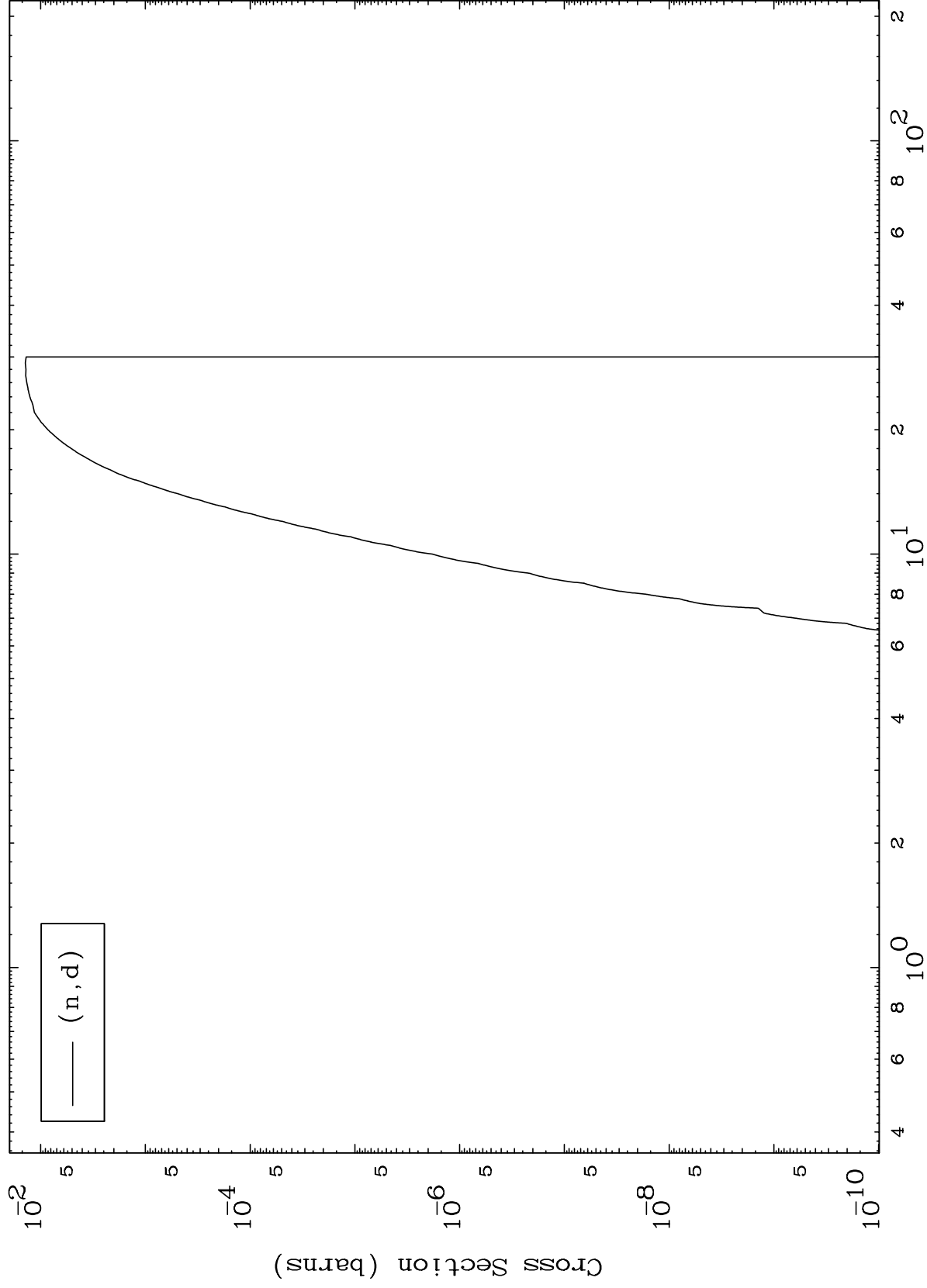
8



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(n,d) Levels
293 Kelvin Cross Sections

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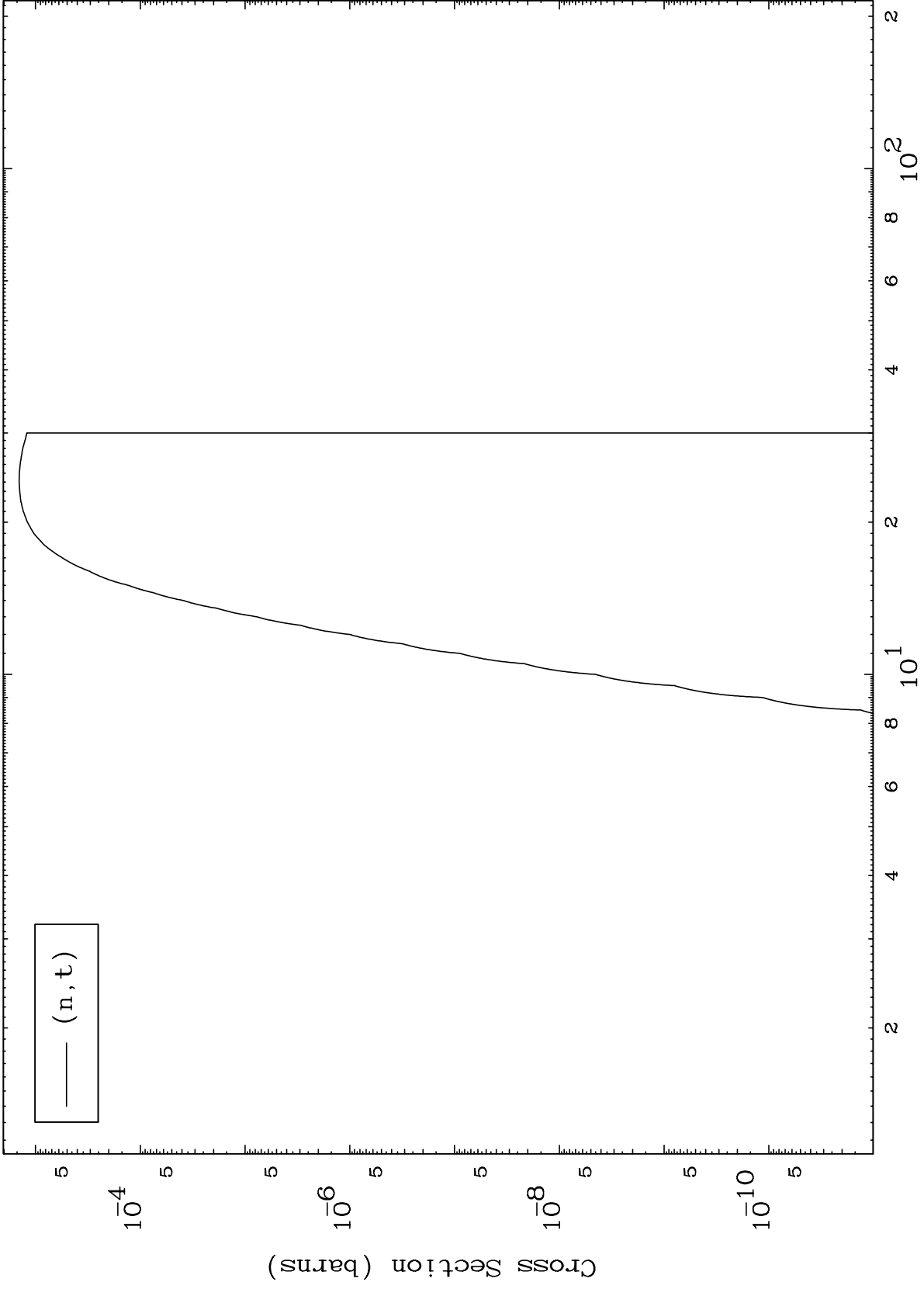
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

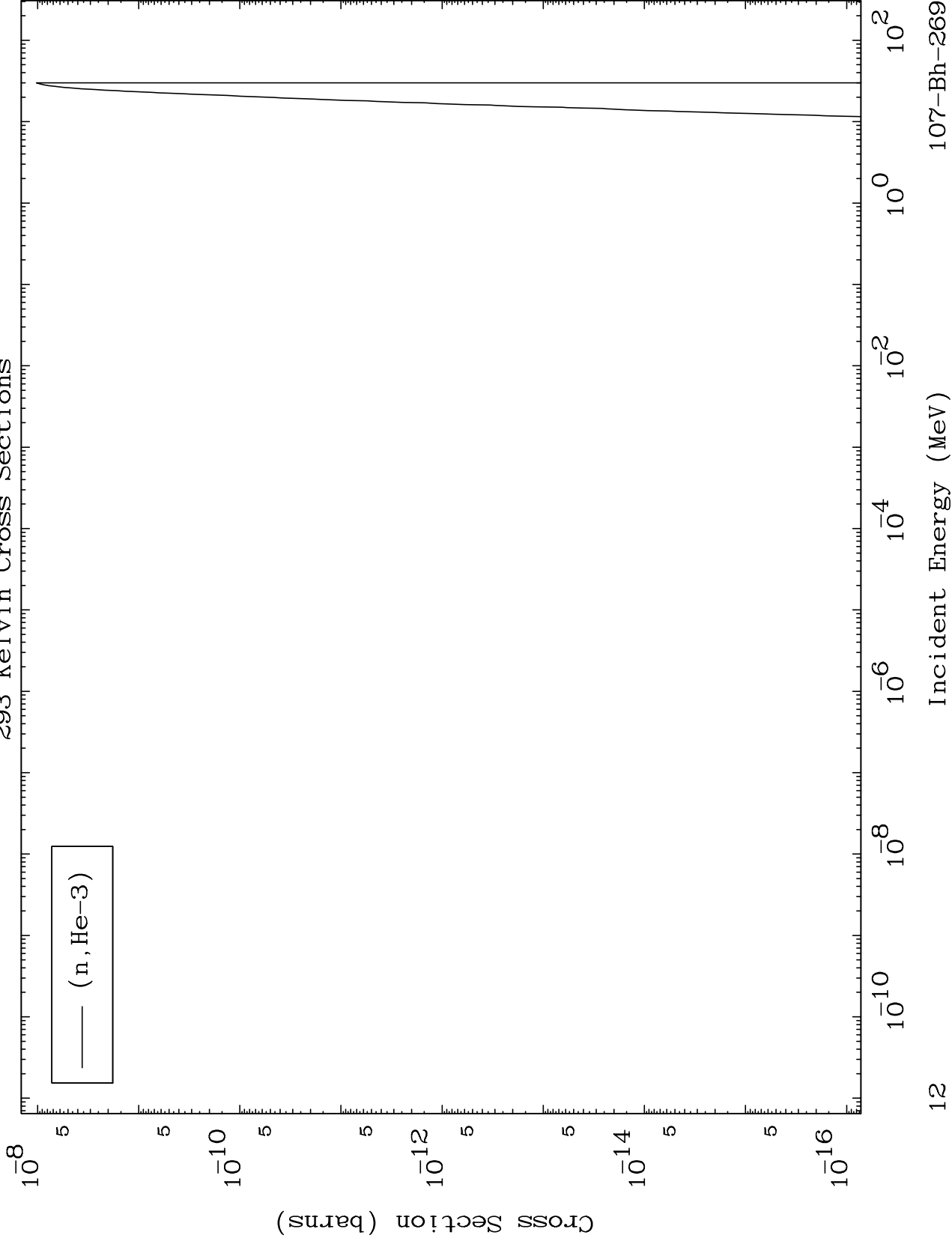
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(n,He3) Levels
293 Kelvin Cross Sections

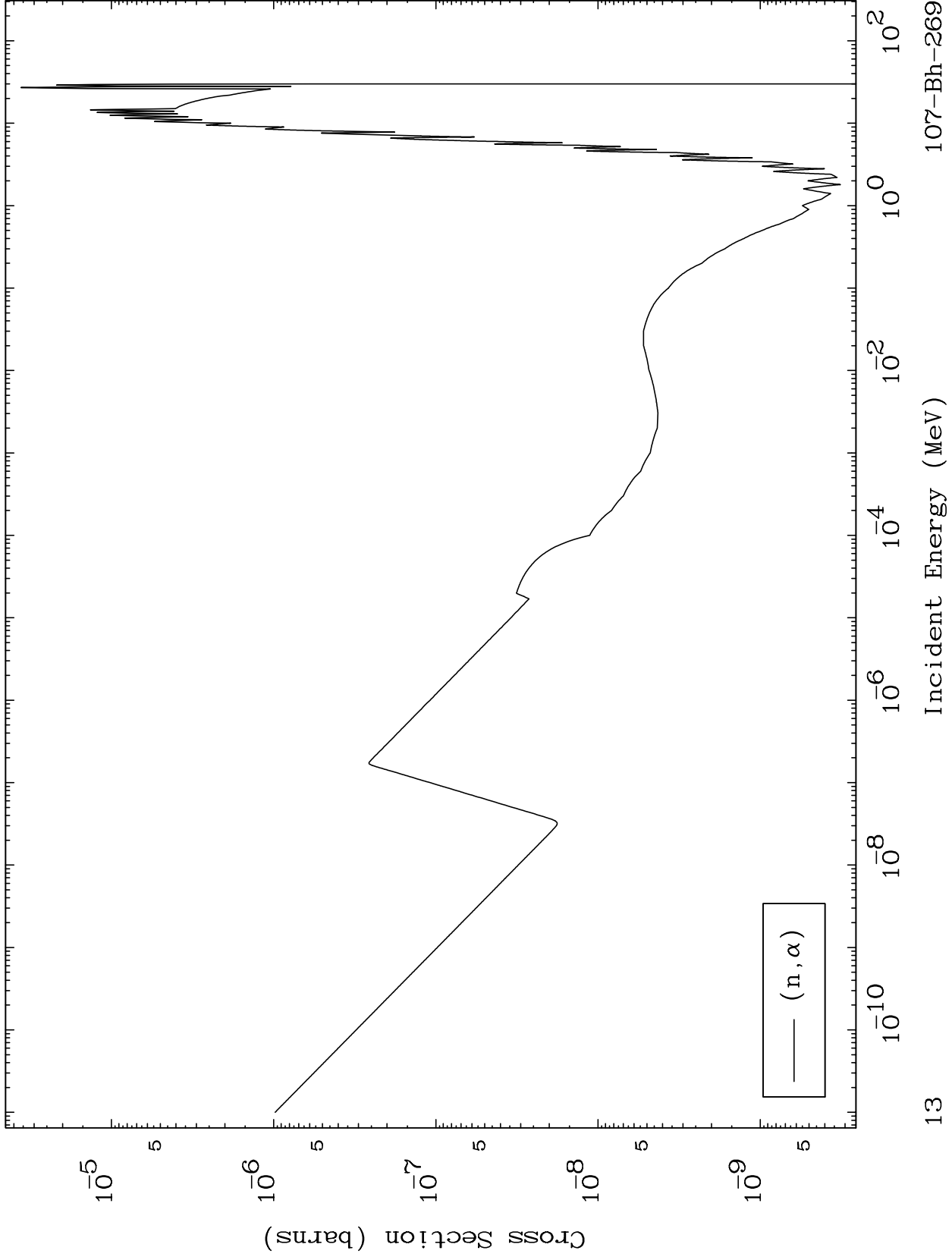
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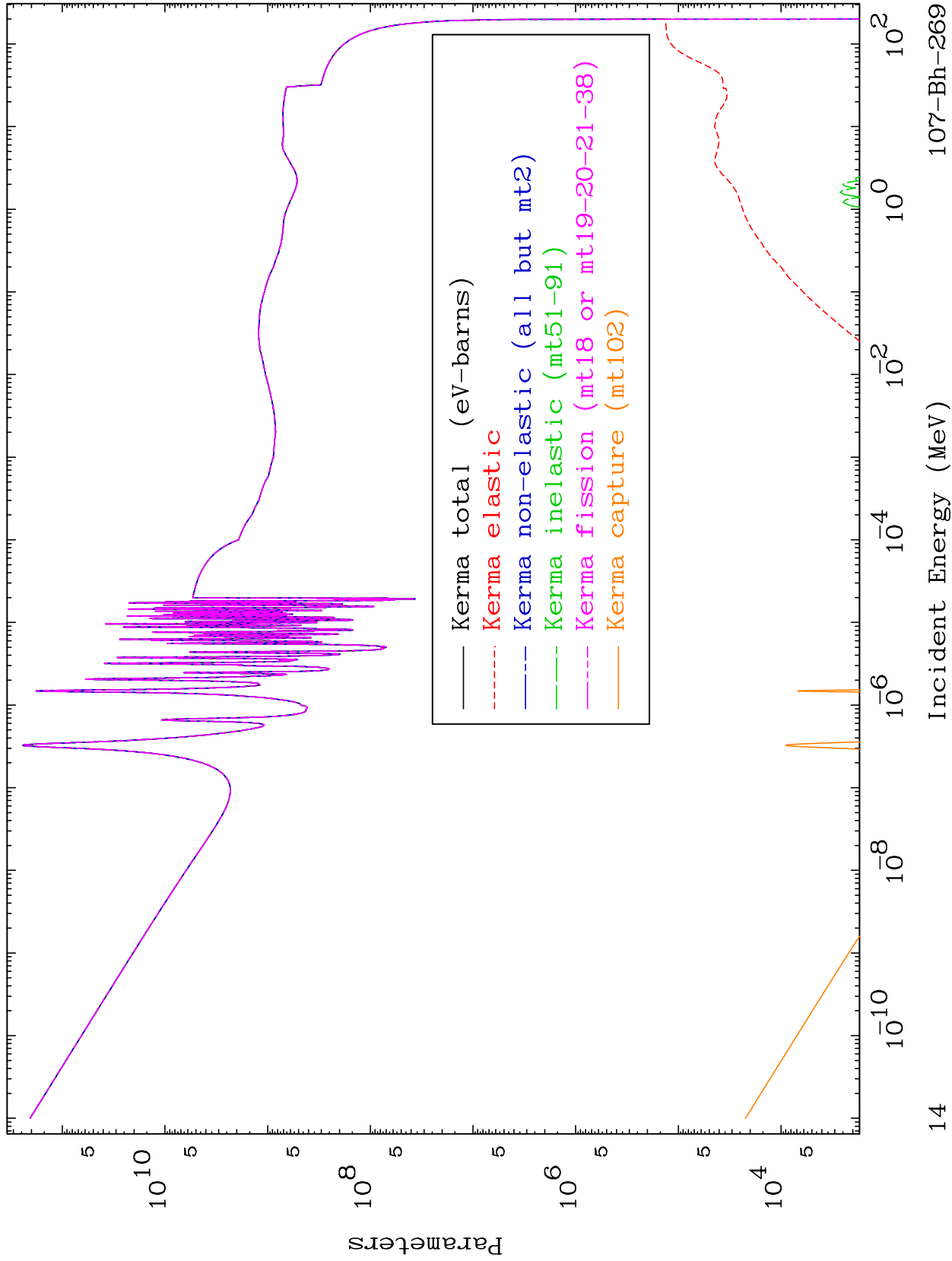


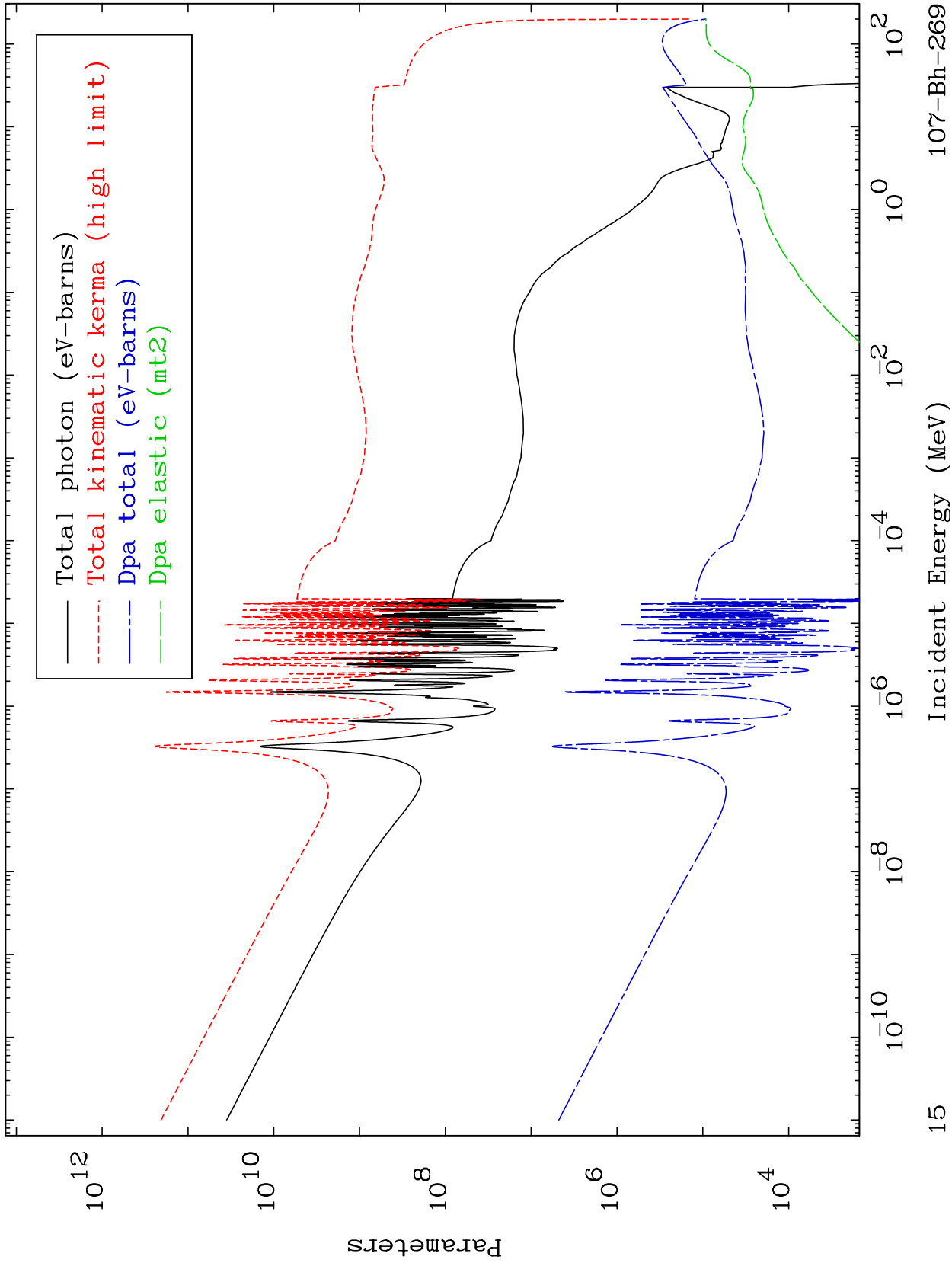
MAT 769

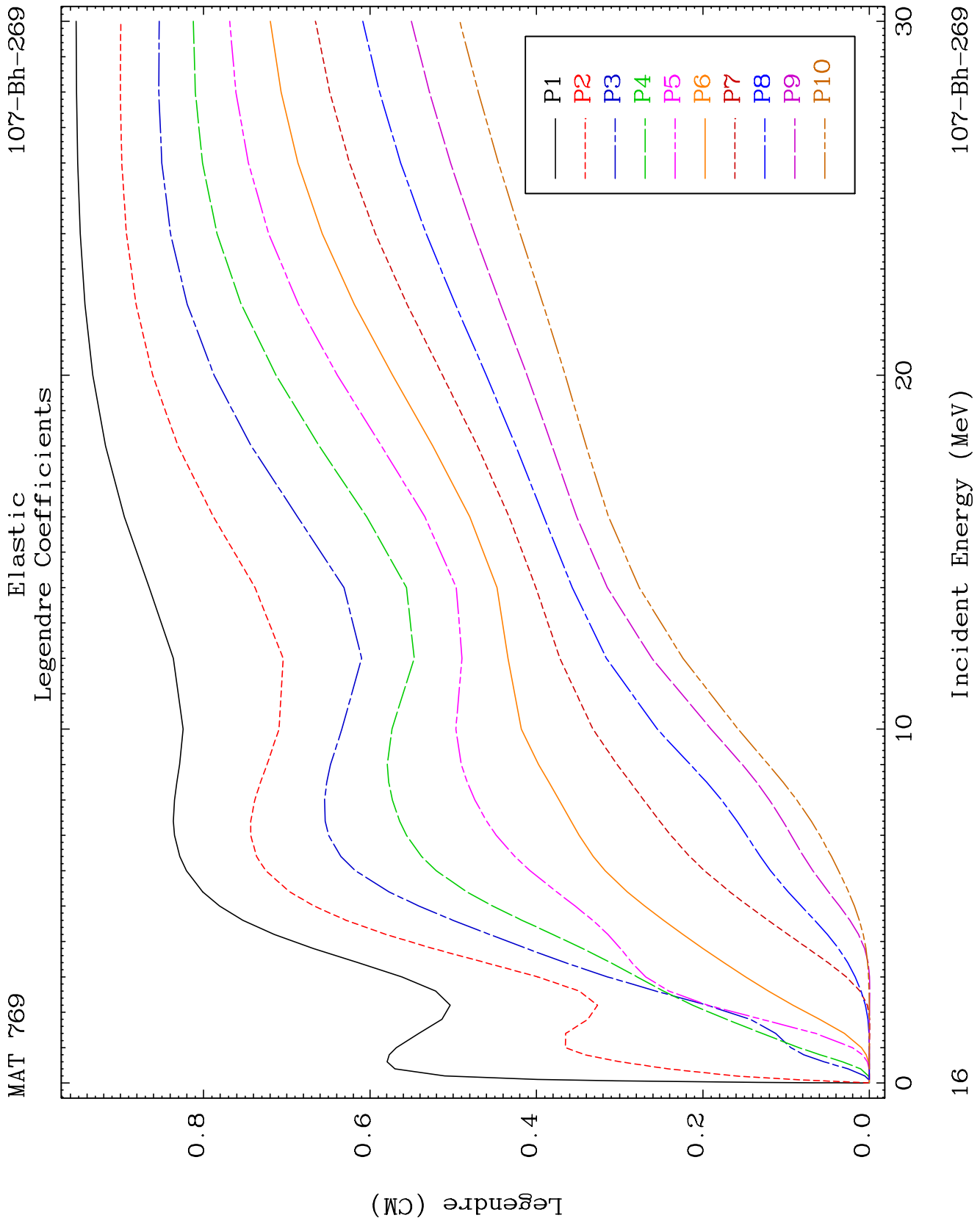
(n, α) Levels
293 Kelvin Cross Sections

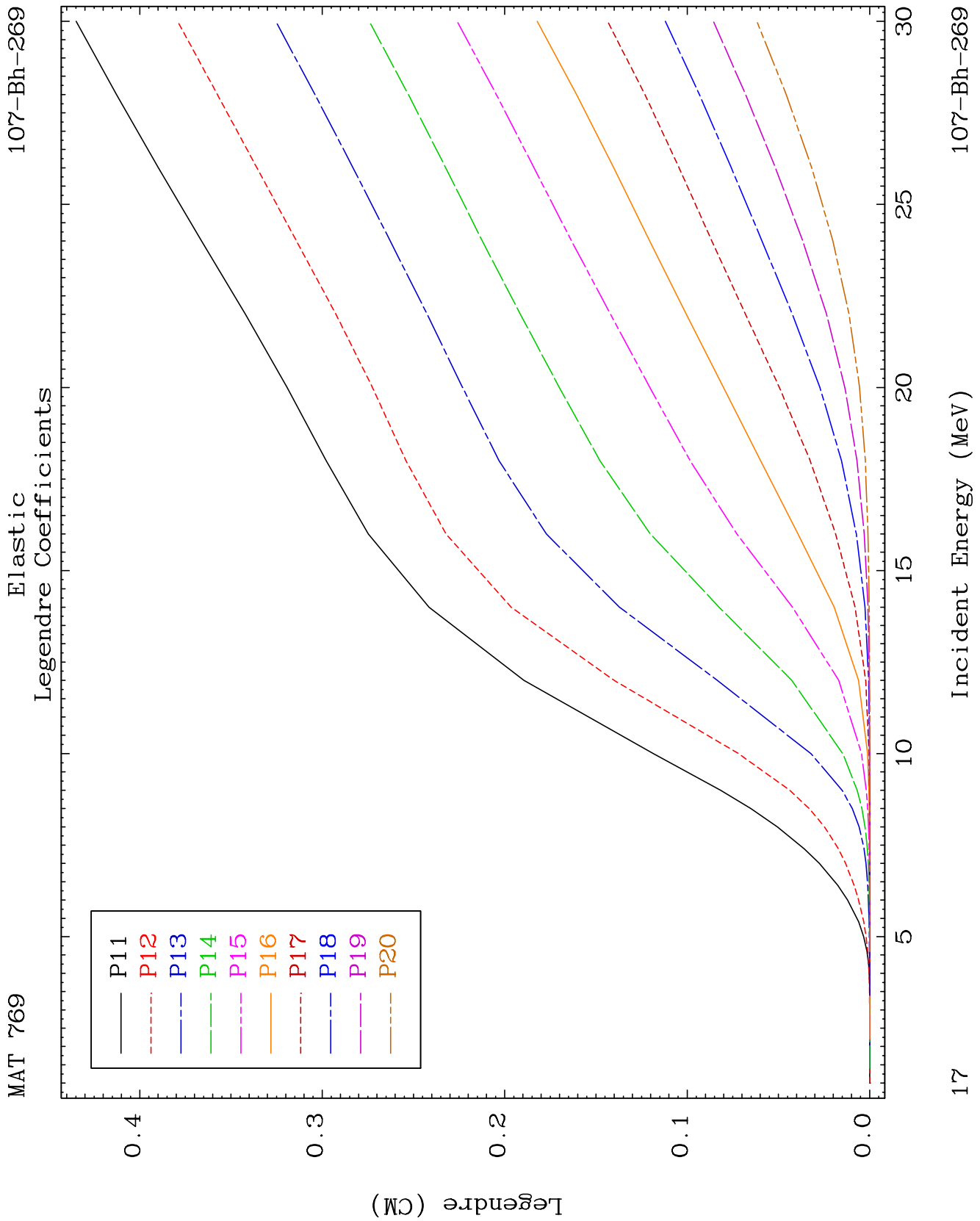
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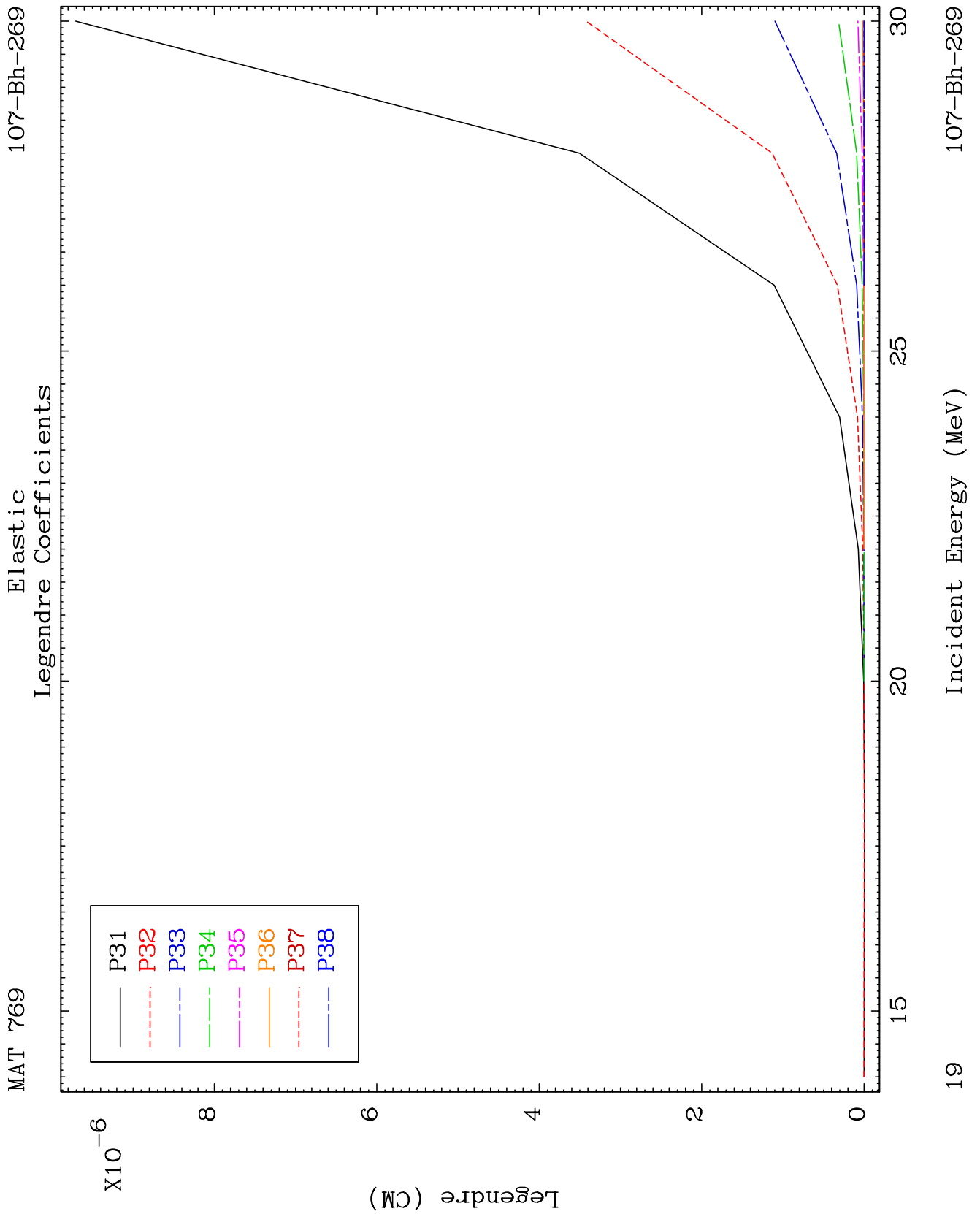








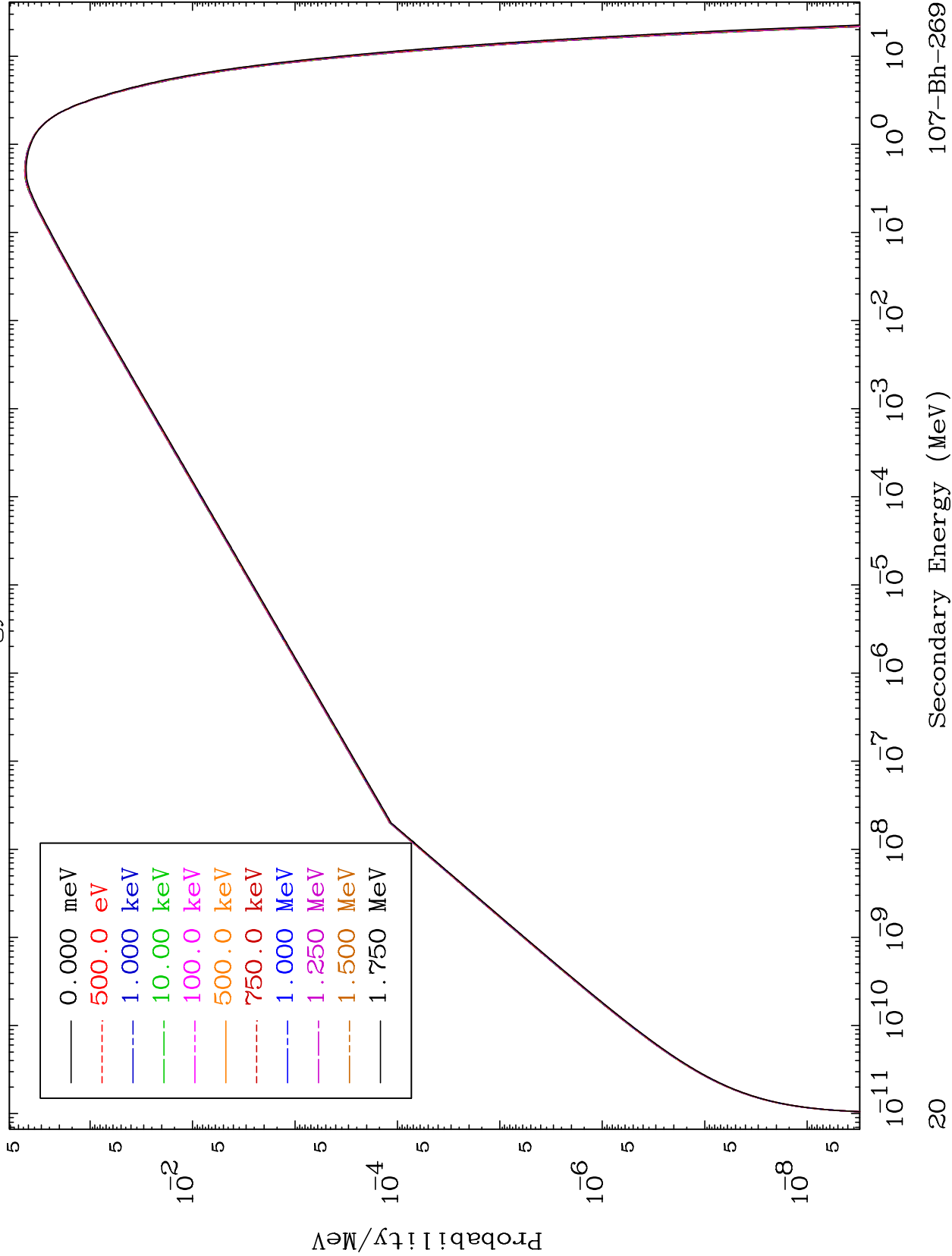




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Fission Energy Distribution

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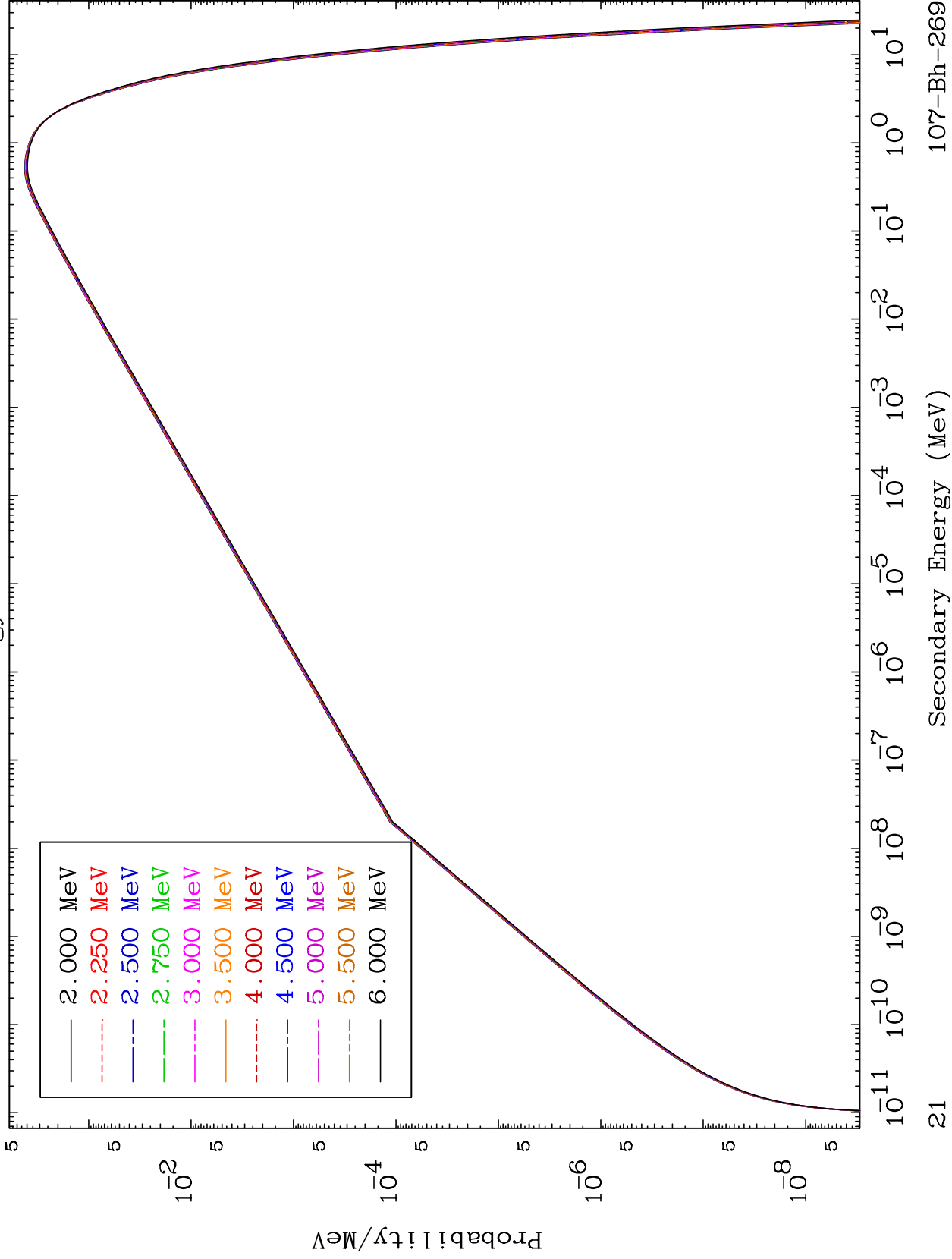
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Secondary Energy (MeV)

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Fission Energy Distribution

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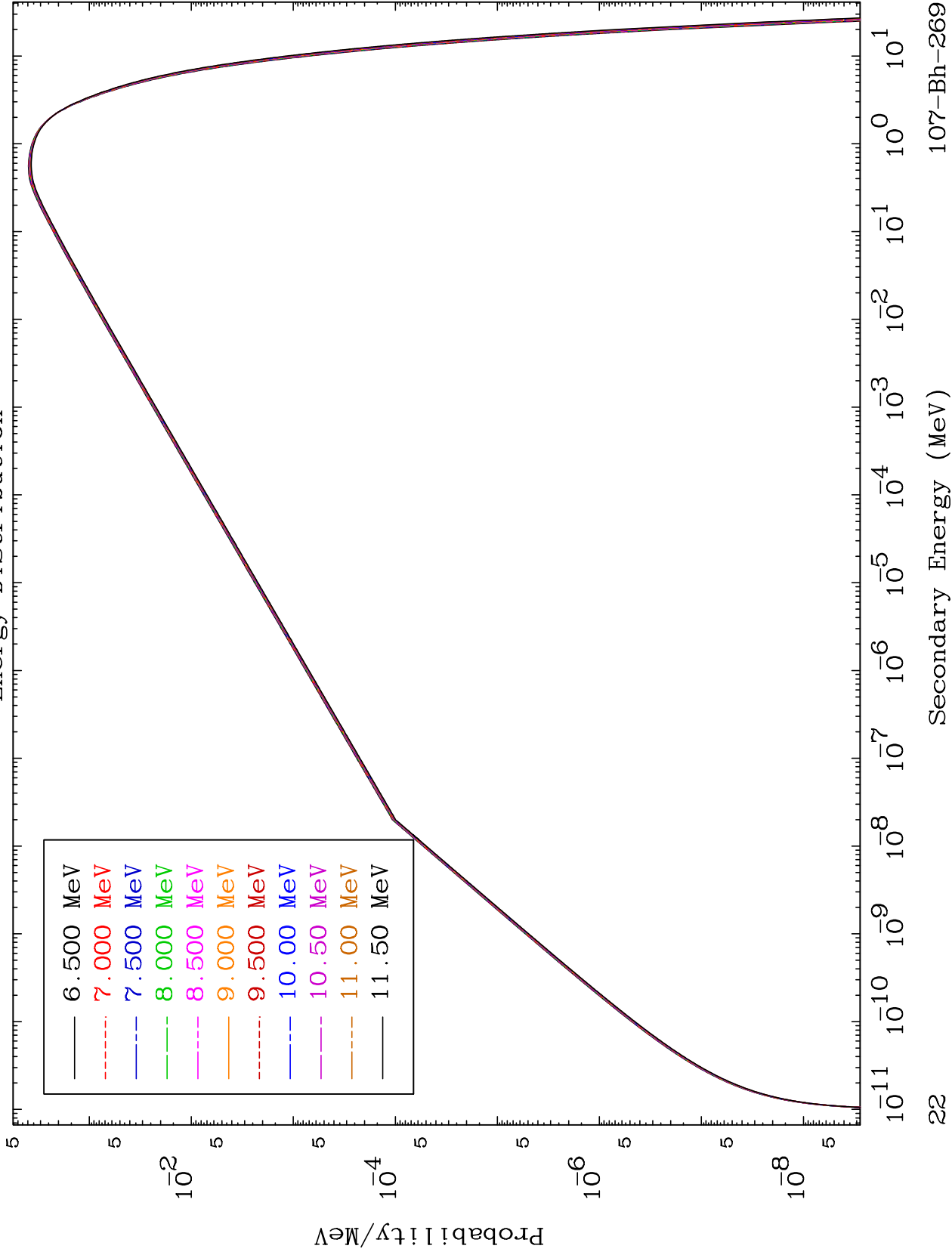


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Fission Energy Distribution

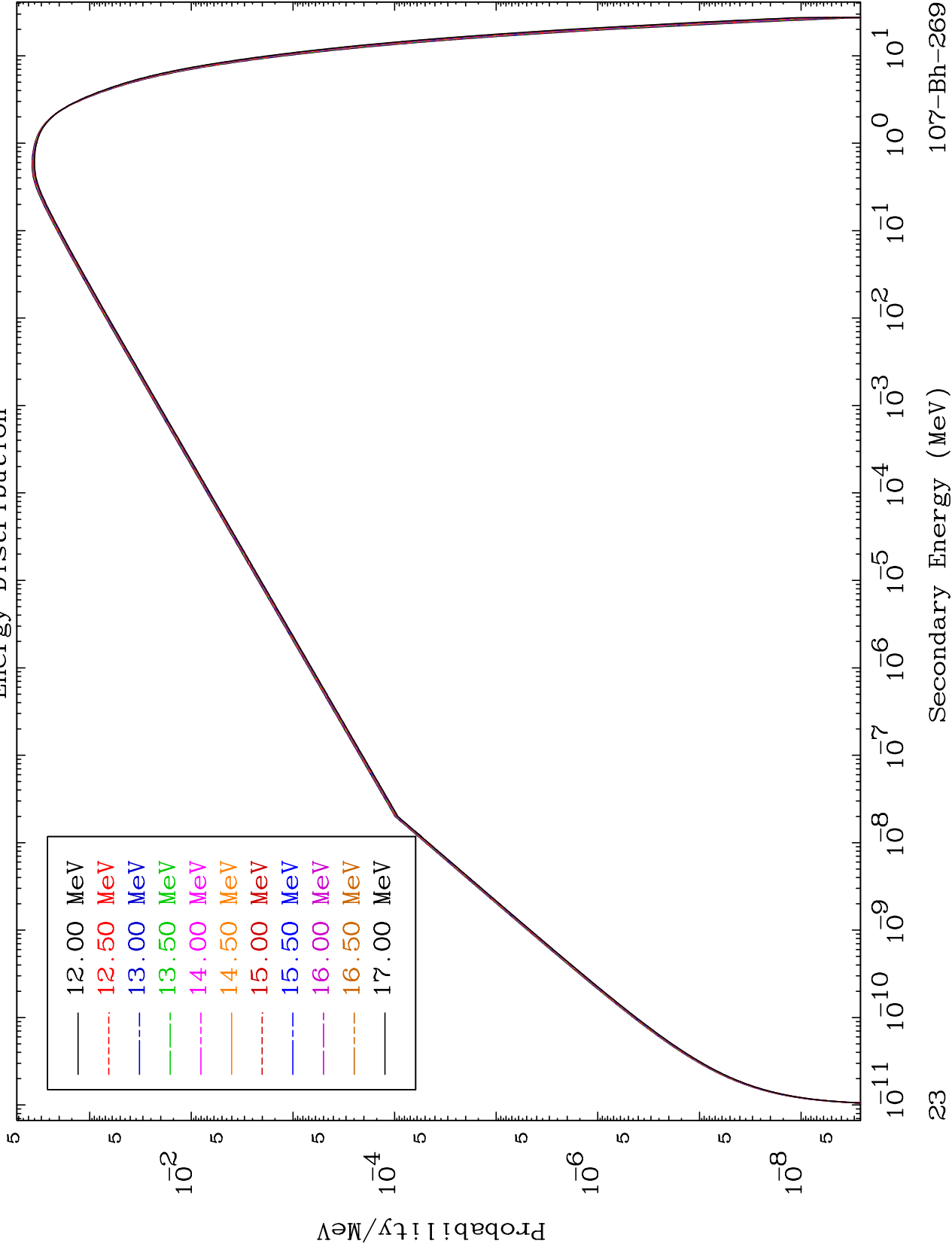
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Fission Energy Distribution

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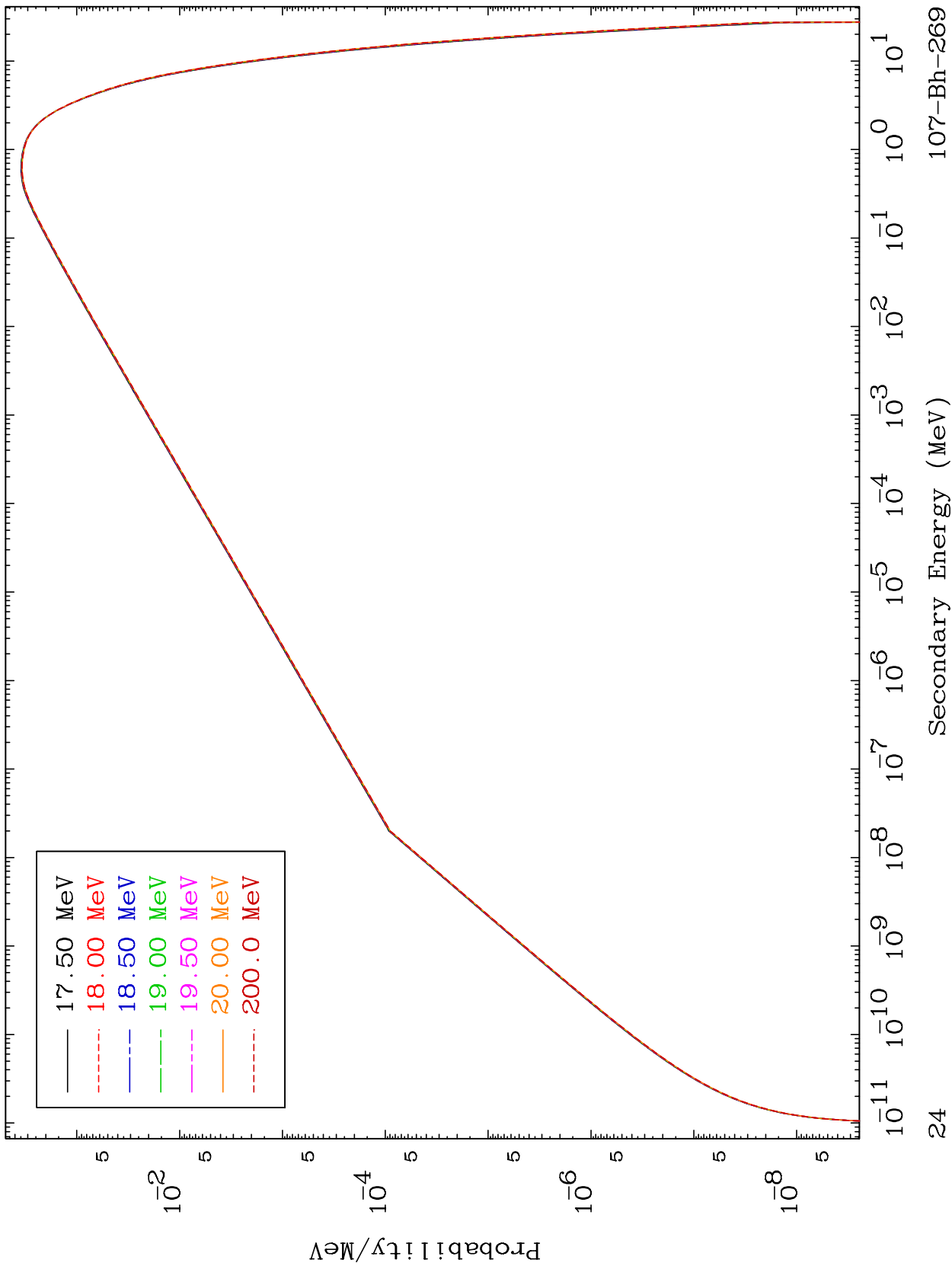
Secondary Energy (MeV)

23

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Fission Energy Distribution

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Secondary Energy (MeV)

24