

Program EVALPLOT
(Version 2018-1)

by

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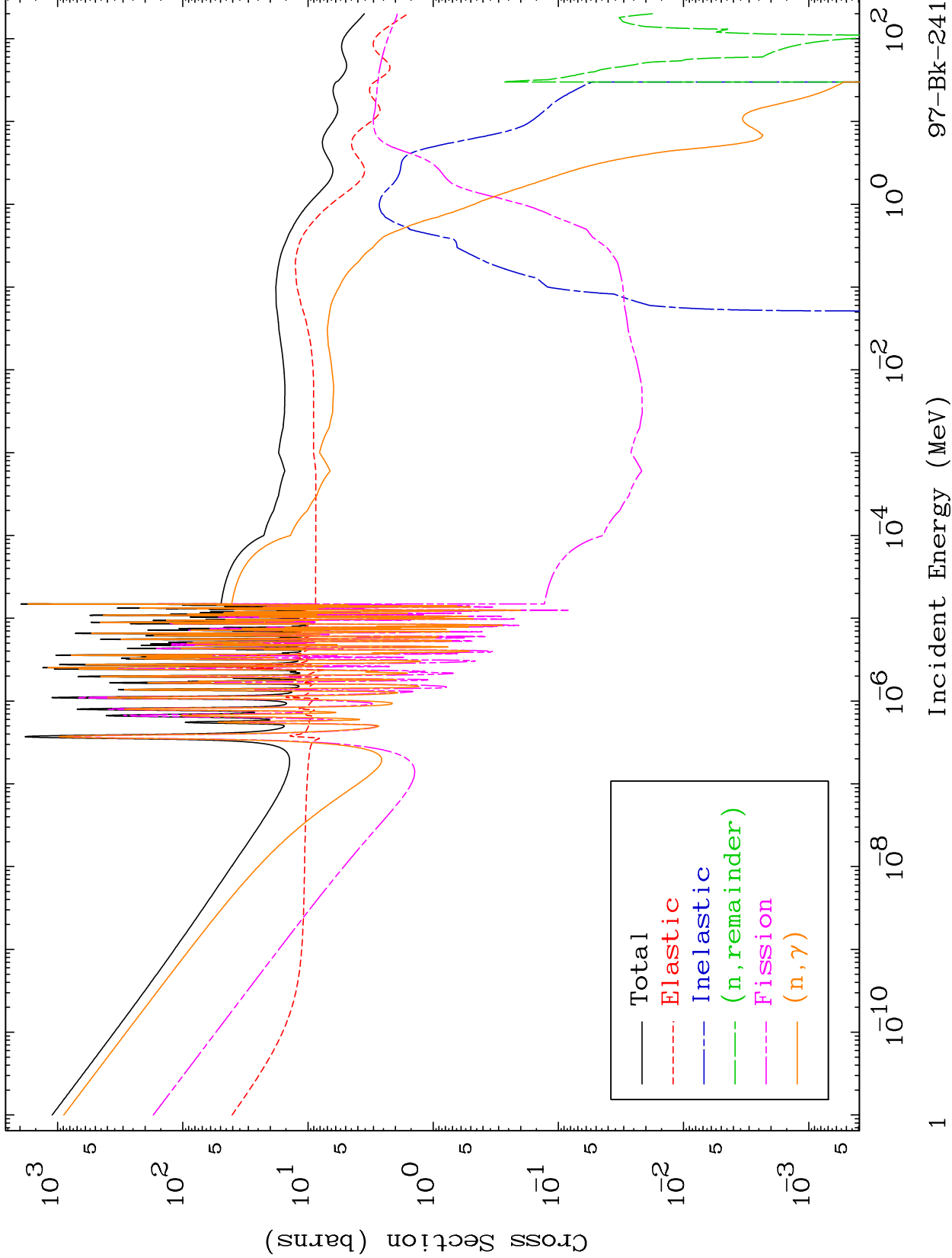
Press Mouse Button to Start

MAT 9728

Major

293 Kelvin Cross Sections

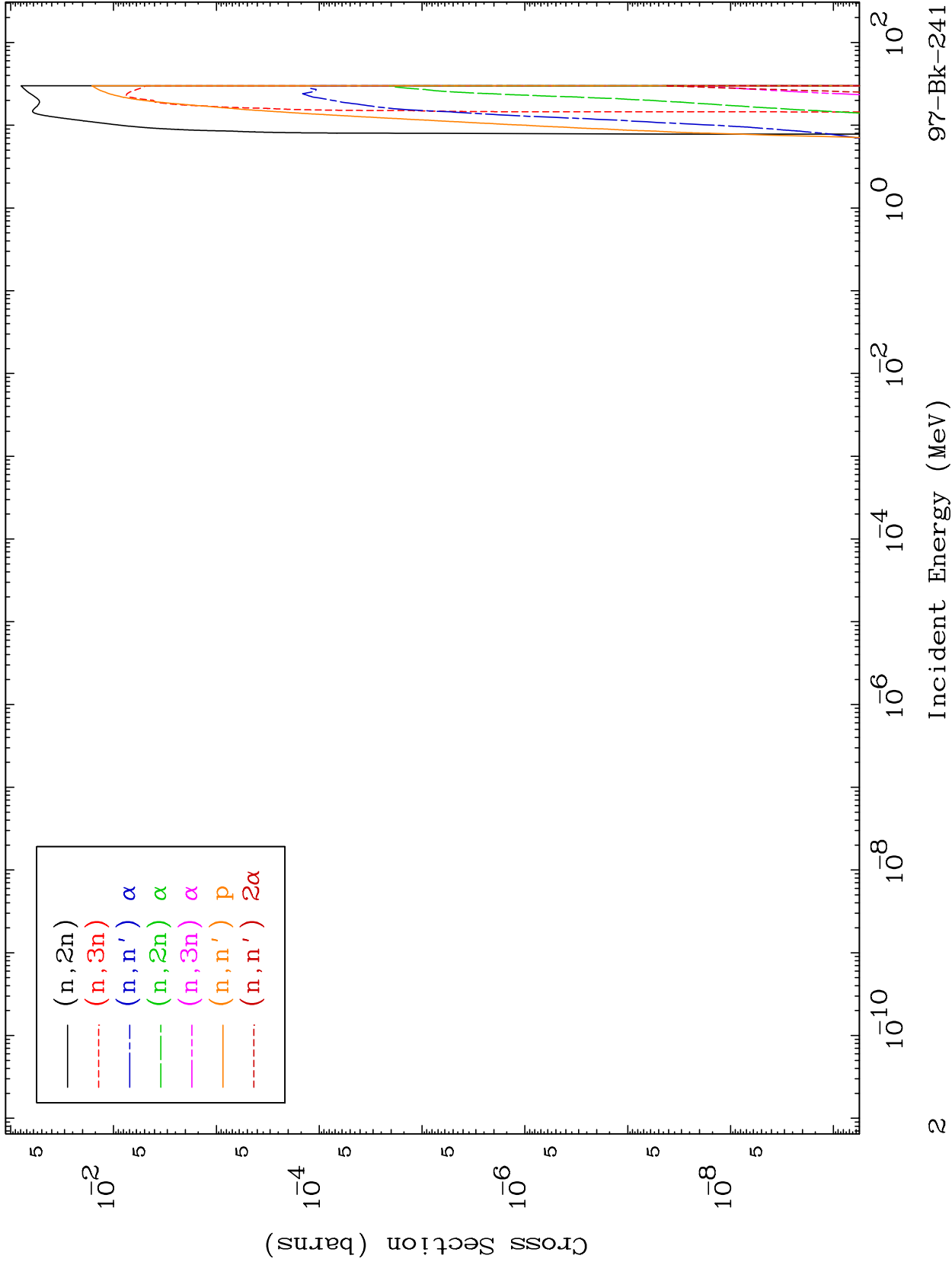
97-Bk-241

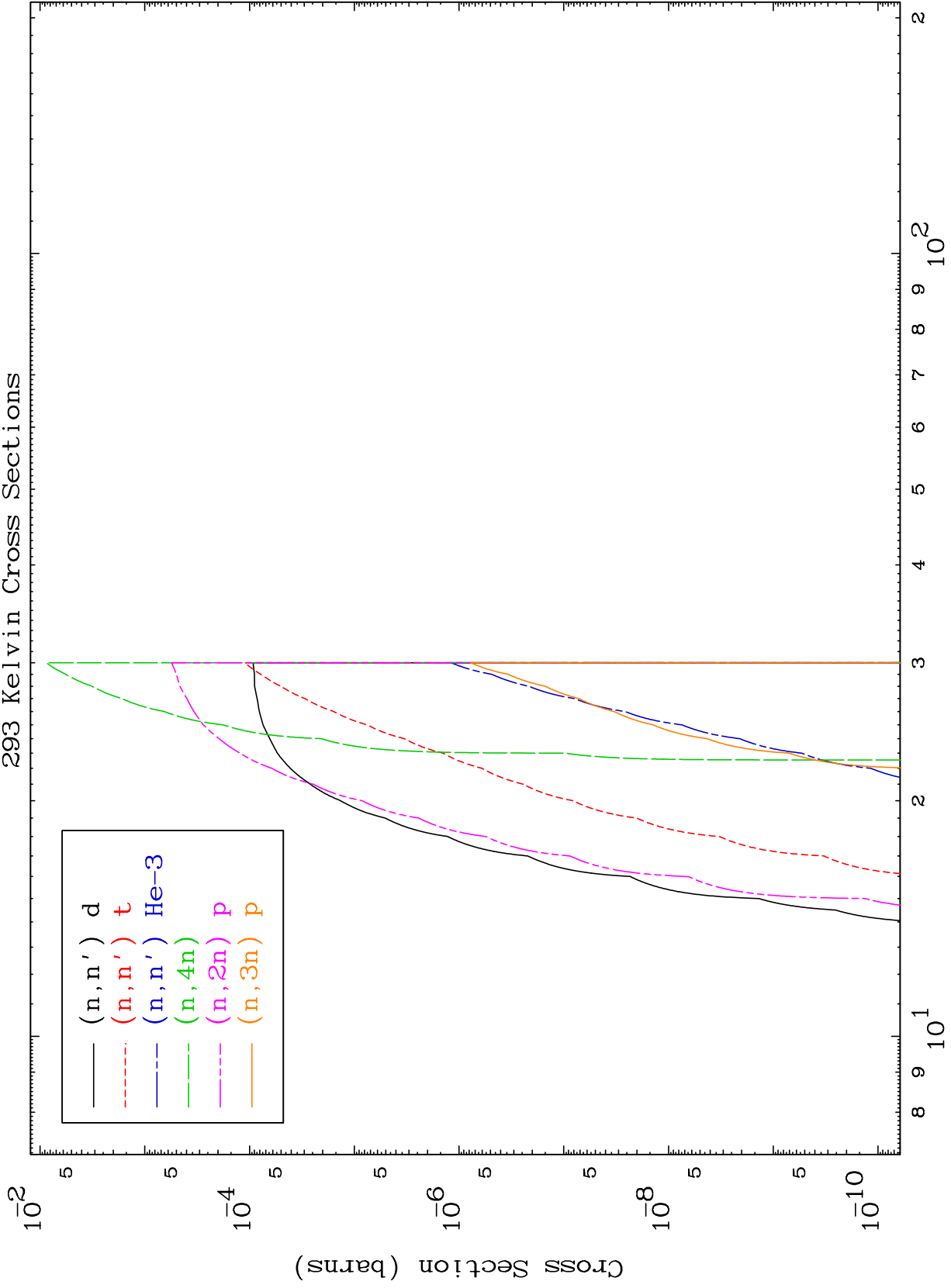


MAT 9728

Neutron Production
293 Kelvin Cross Sections

97-Bk-241

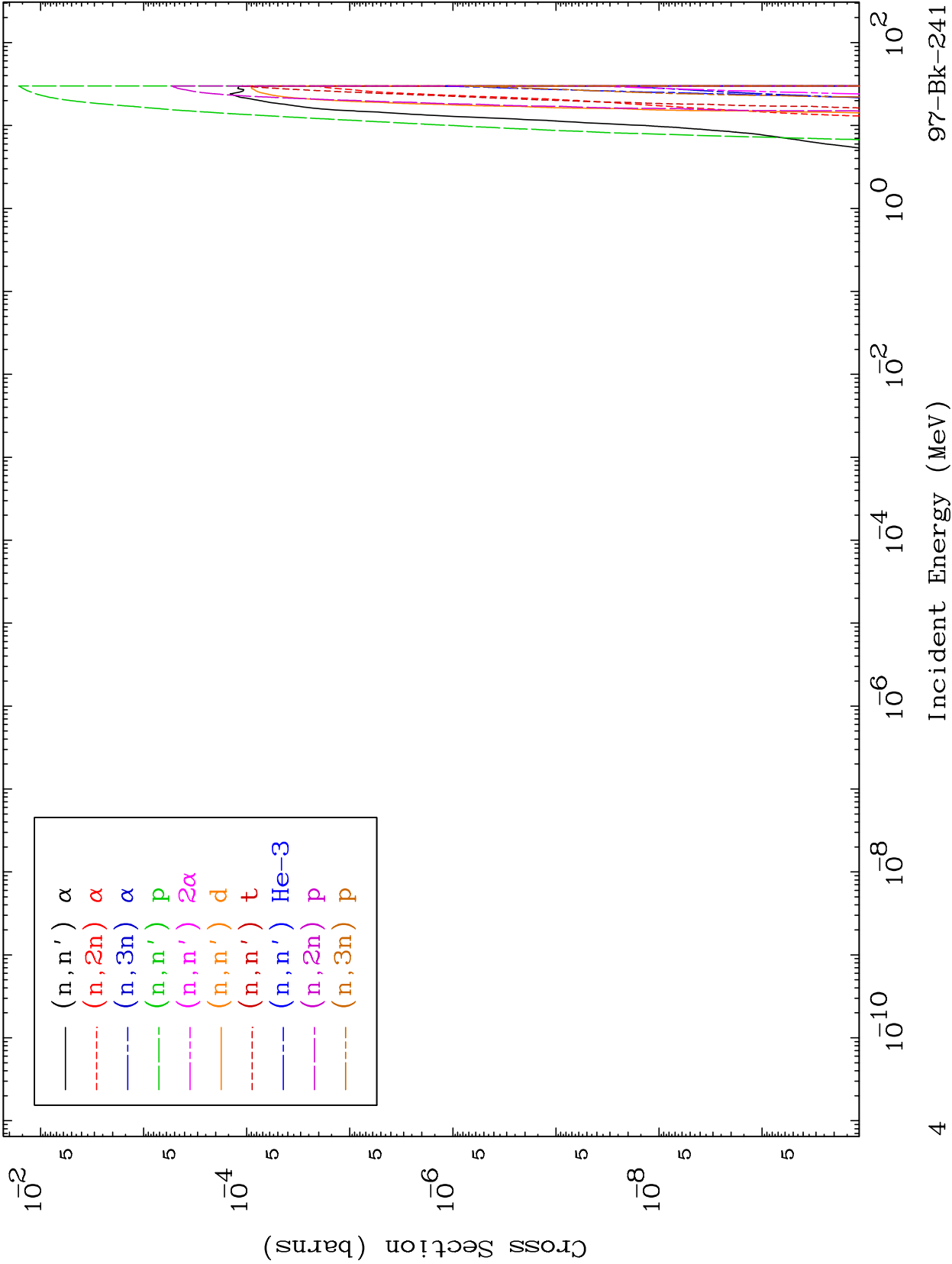




MAT 9728

Charged Particle
293 Kelvin Cross Sections

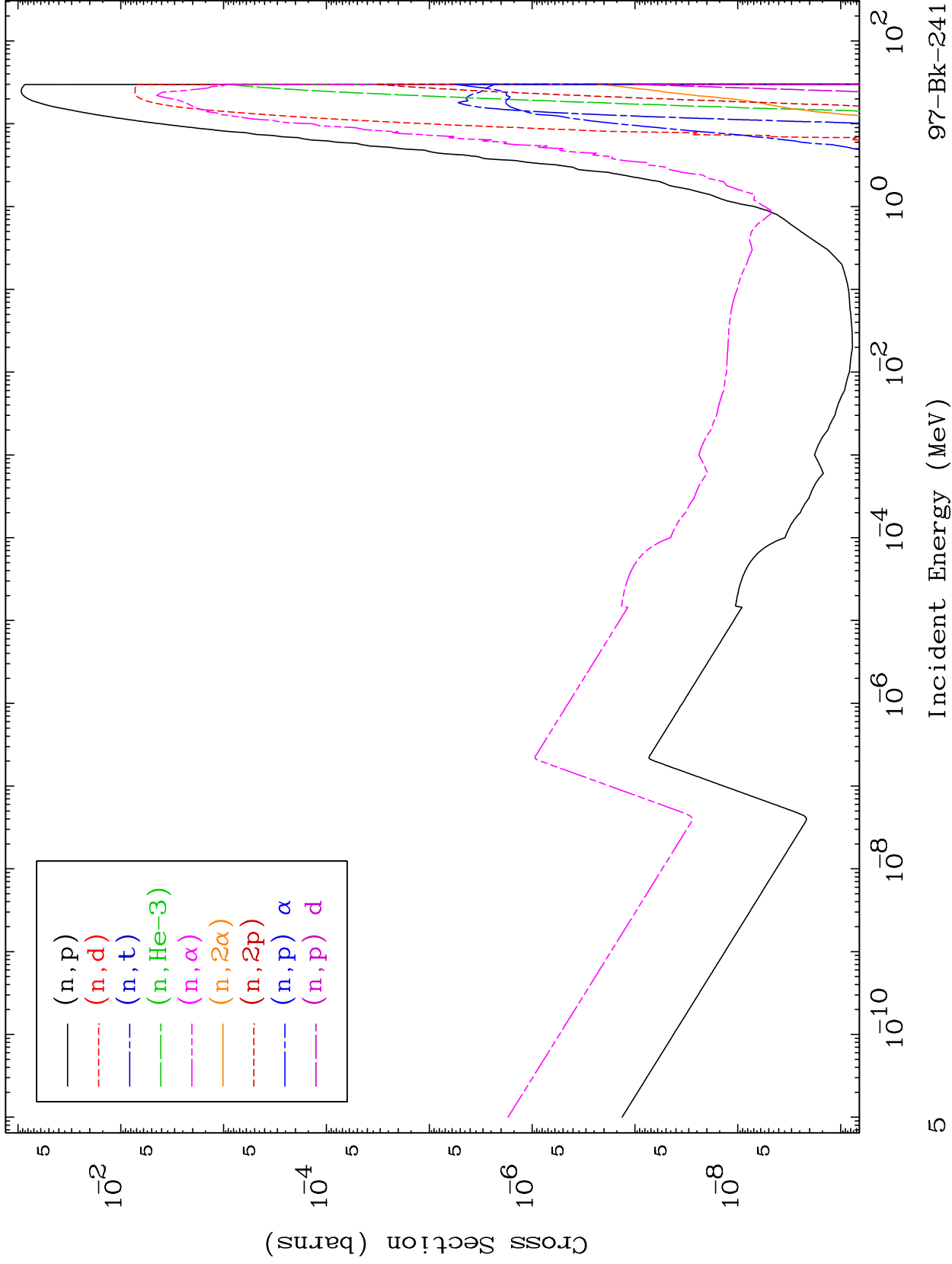
97-Bk-241



MAT 9728

Charged Particle
293 Kelvin Cross Sections

97-Bk-241



97-Bk-241

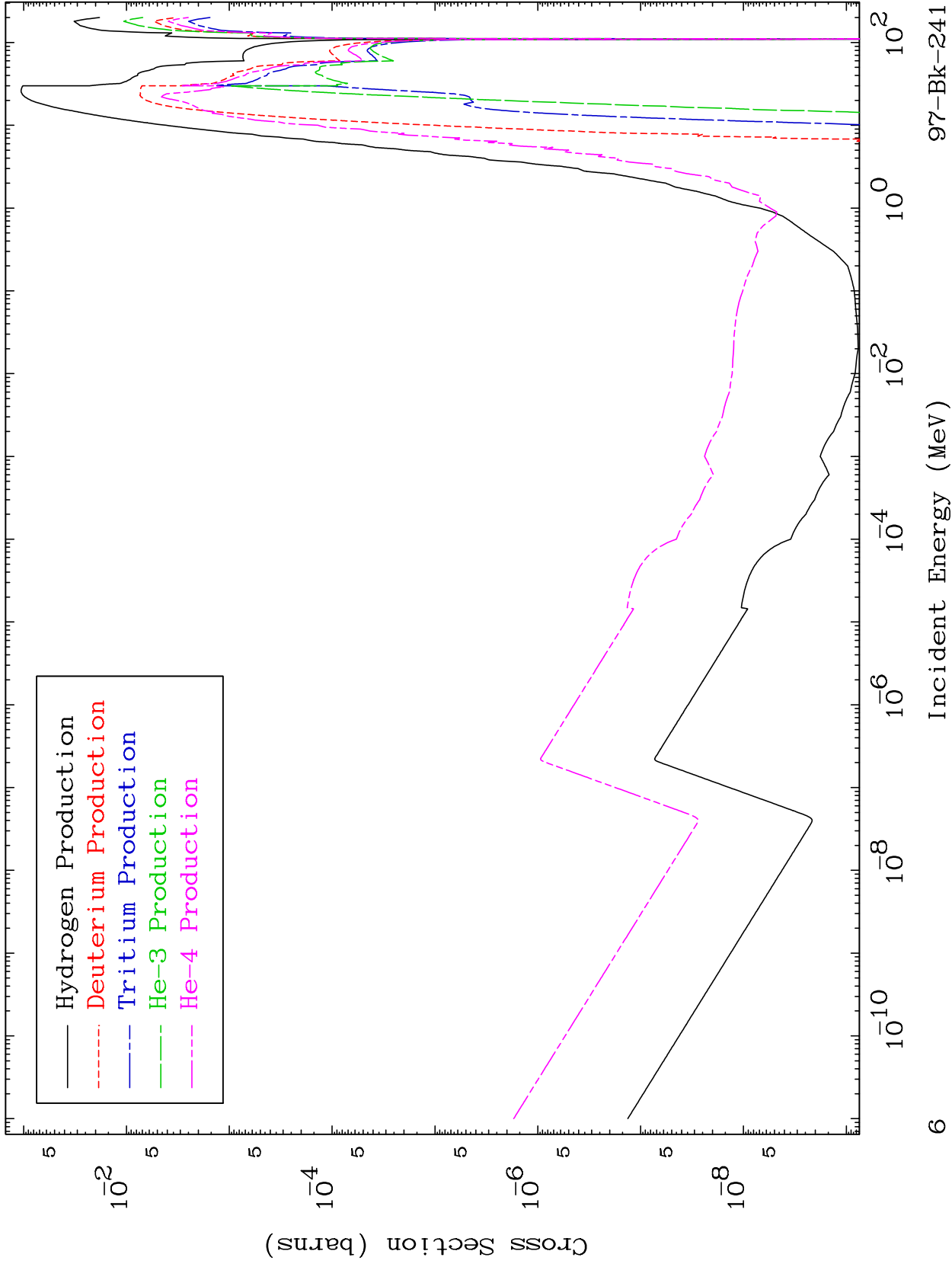
Incident Energy (MeV)

5

MAT 9728

Particle Production
293 Kelvin Cross Sections

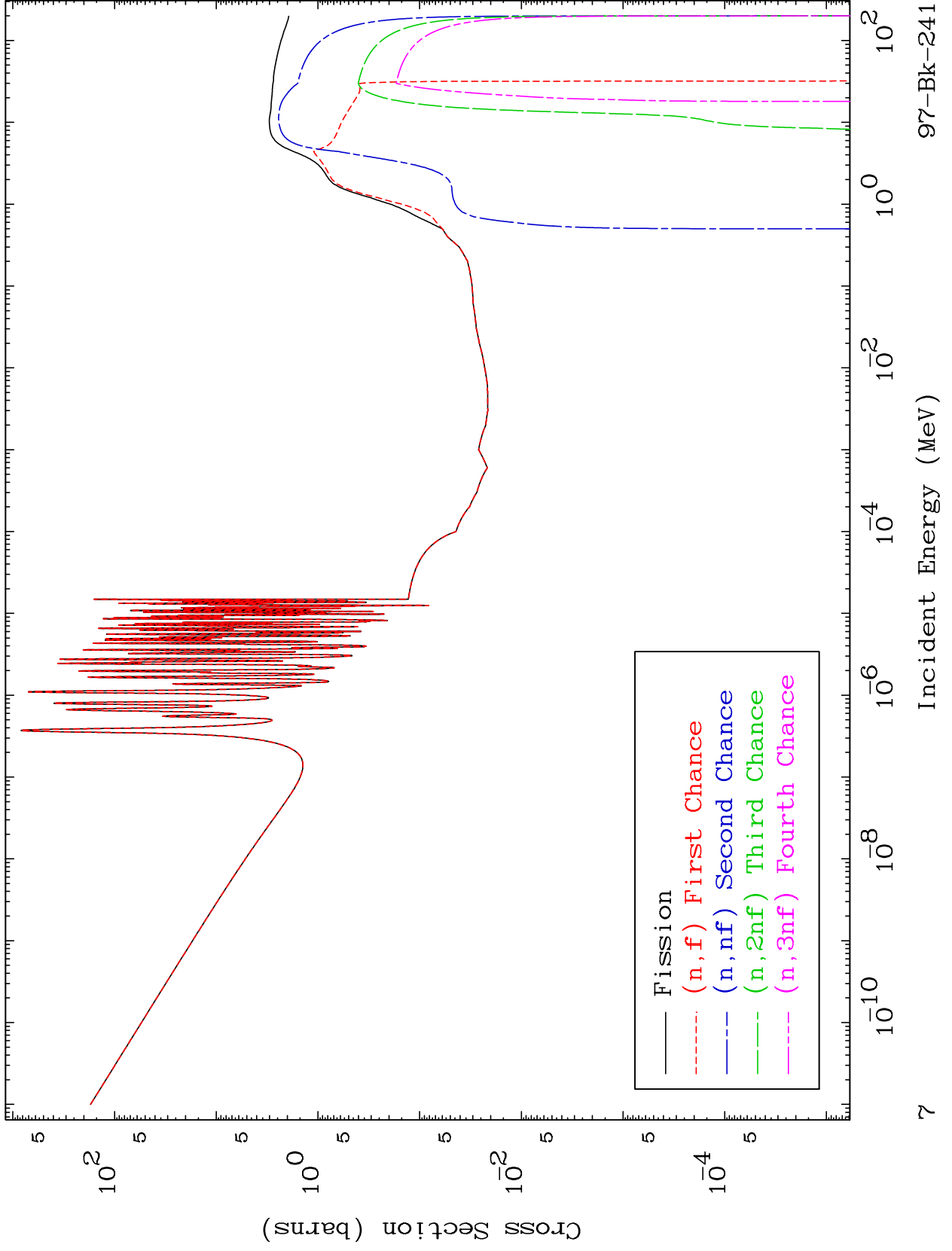
97-Bk-241



MAT 9728

Fission
293 Kelvin Cross Sections

97-Bk-241

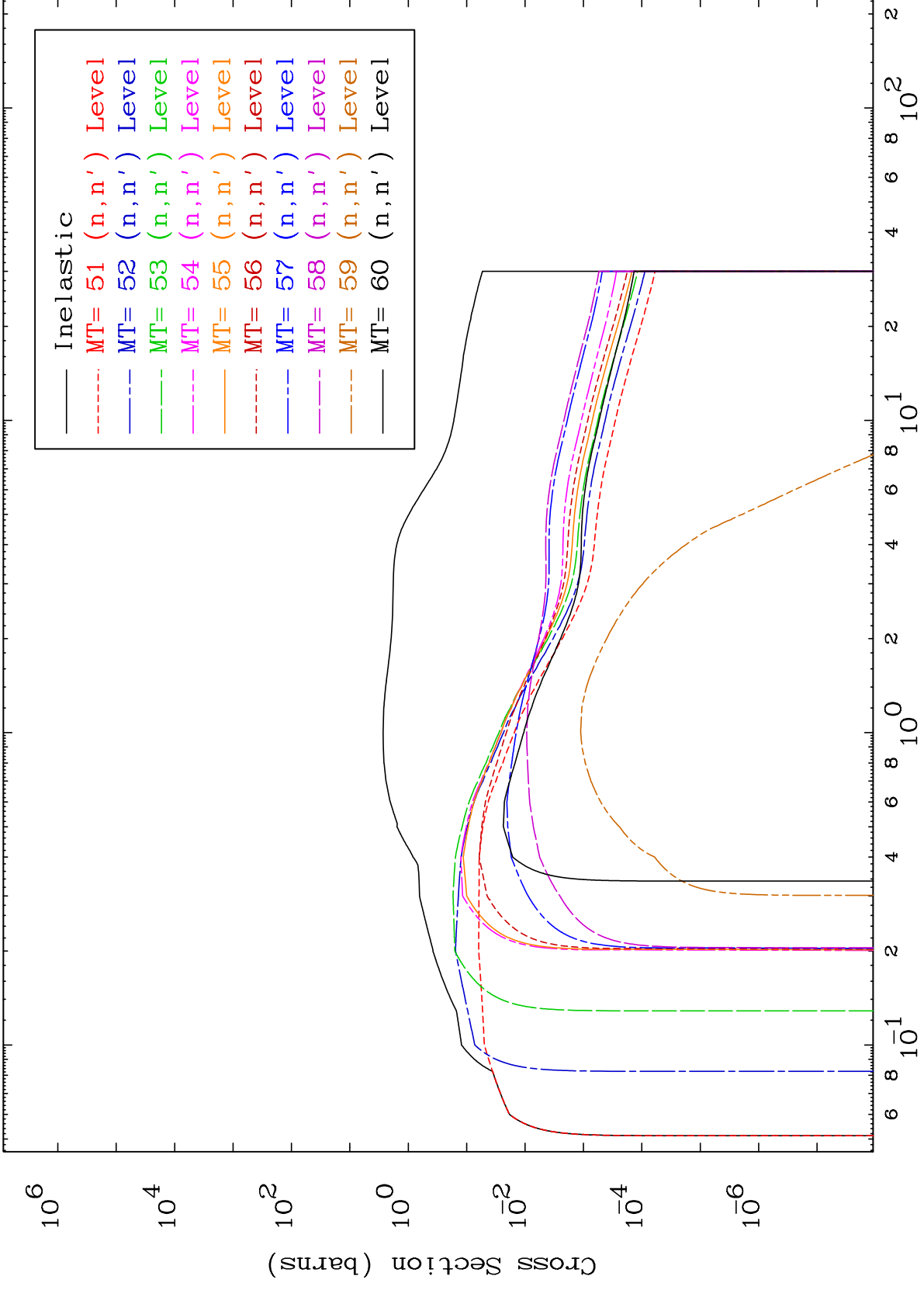


MAT 9728

(n,n') Level

97-Bk-241

293 Kelvin Cross Sections

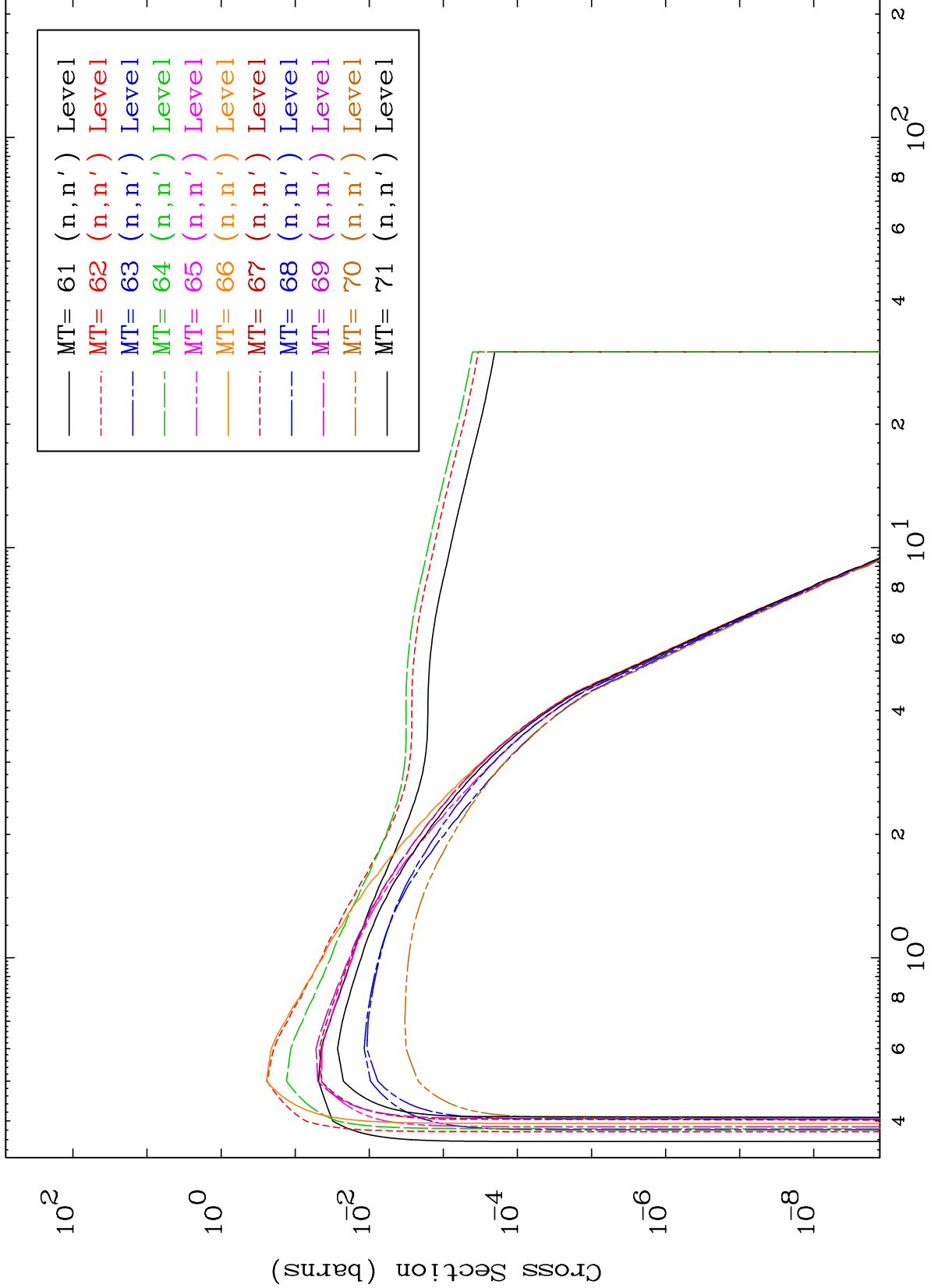


MAT 9728

(n,n') Level

293 Kelvin Cross Sections

97-Bk-241

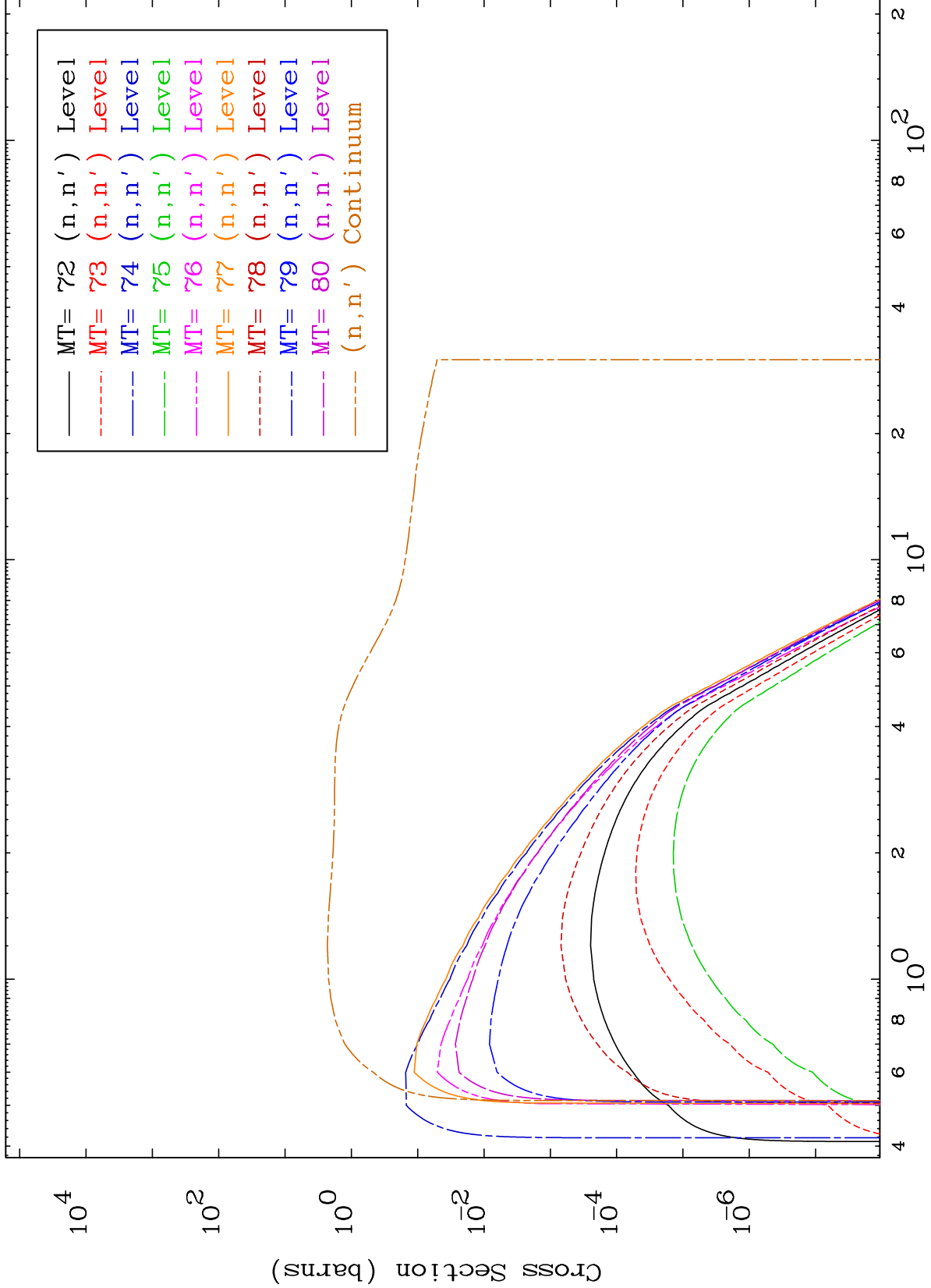


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(n,n') Level

97-Bk-241

293 Kelvin Cross Sections



10

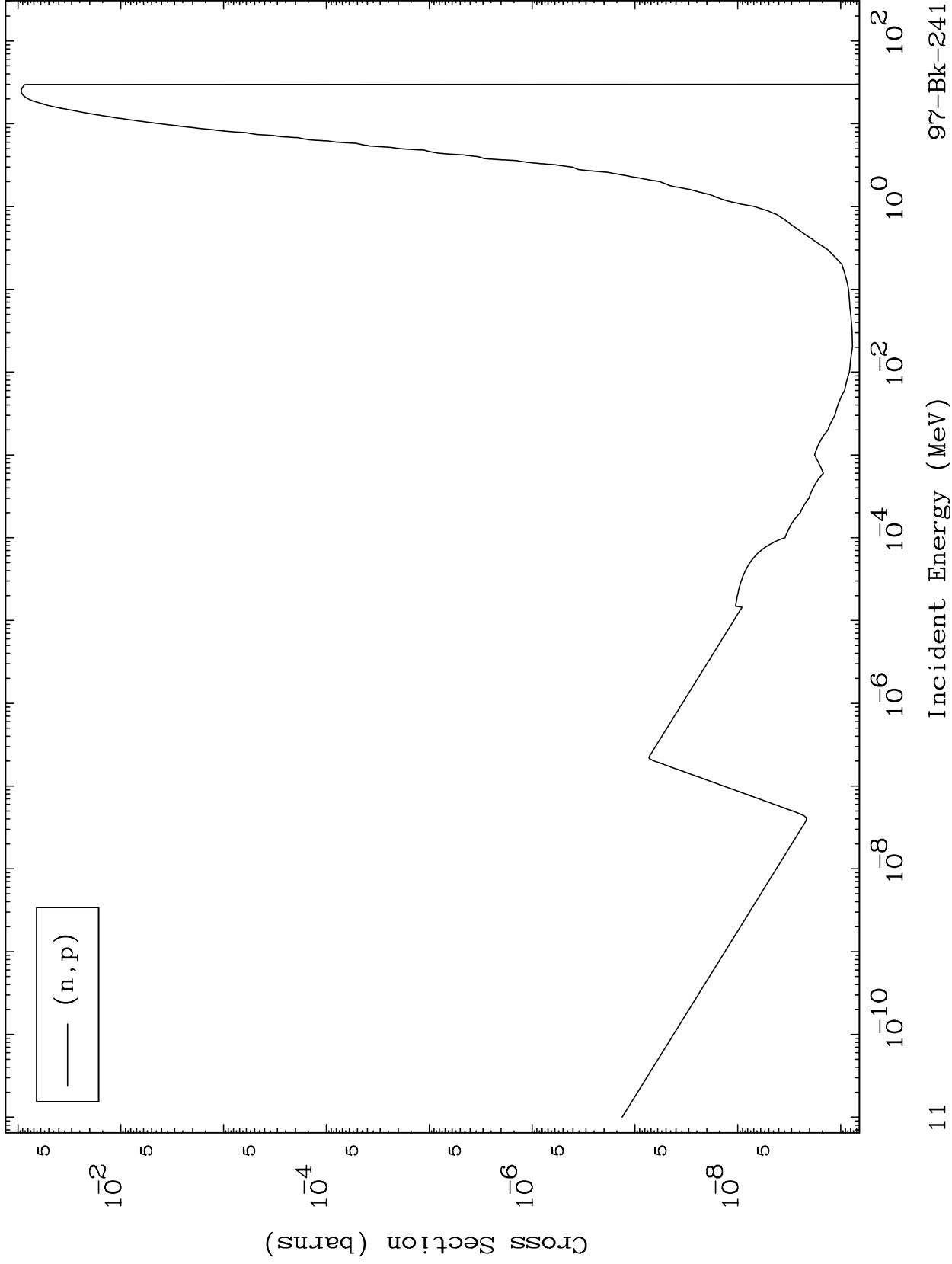
Incident Energy (MeV)

97-Bk-241

MAT 9728

(n,p) Levels
293 Kelvin Cross Sections

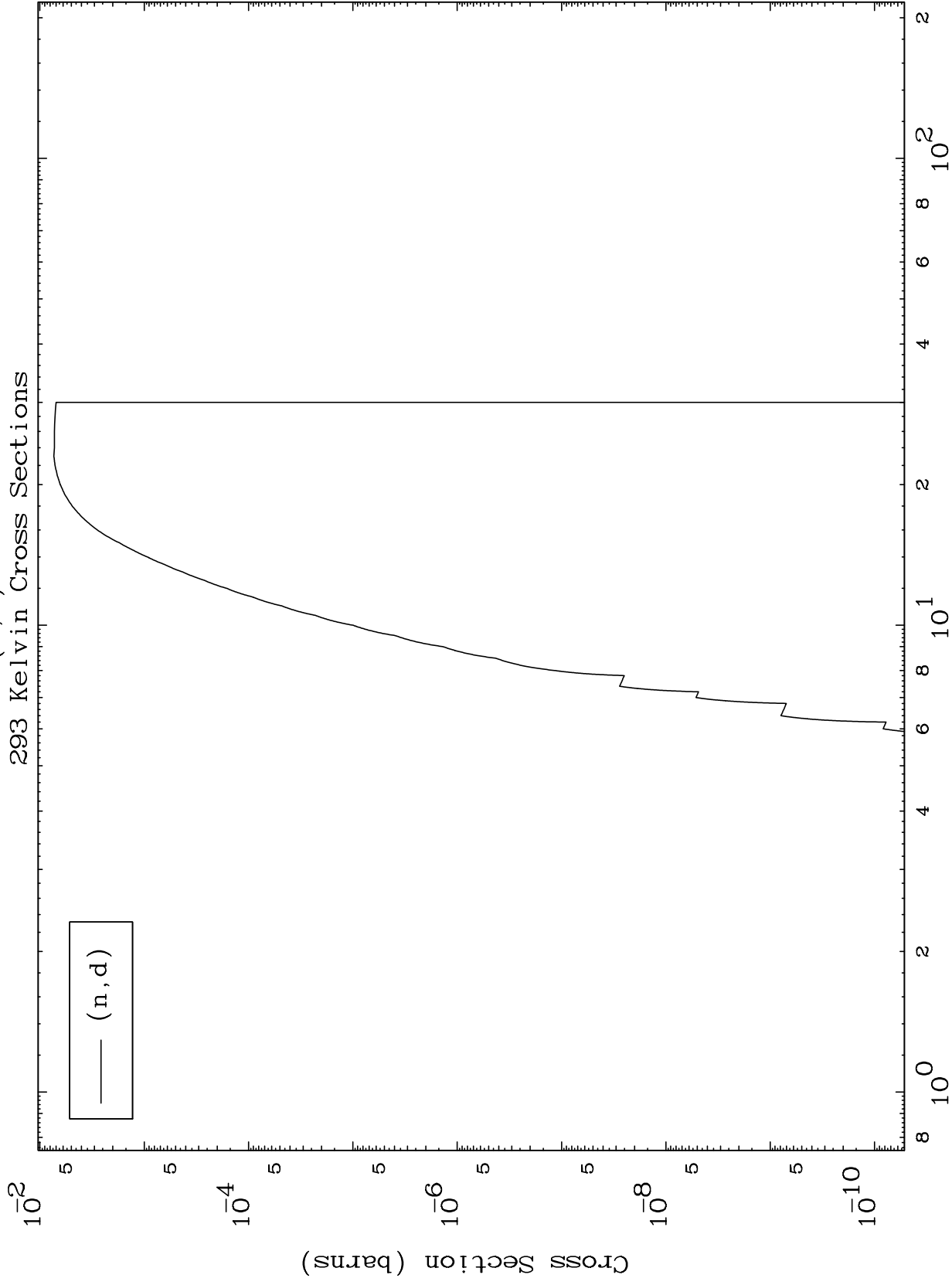
97-Bk-241



MAT 9728

(n,d) Levels
293 Kelvin Cross Sections

97-Bk-241



12

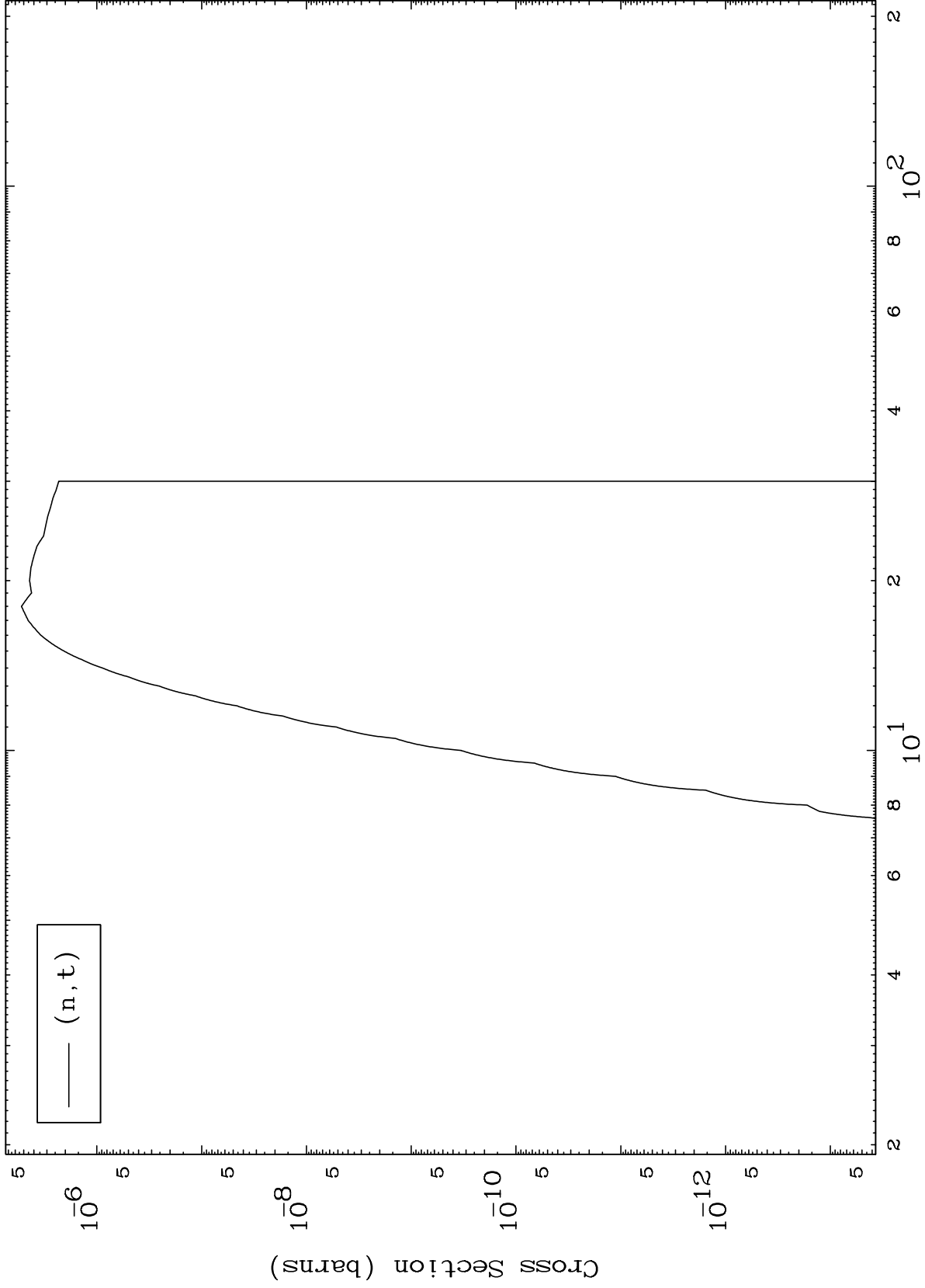
Incident Energy (MeV)

97-Bk-241

MAT 9728

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-241



13

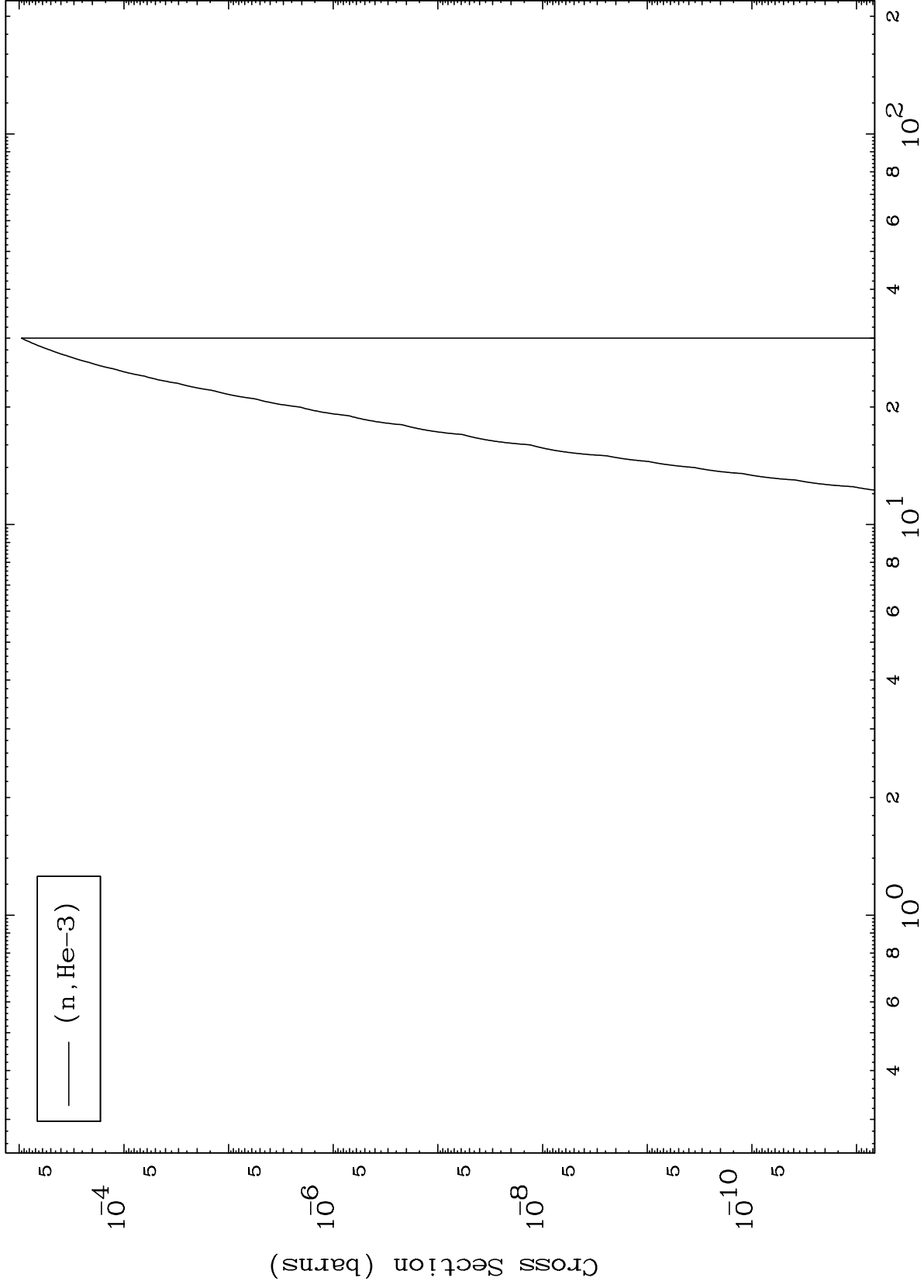
Incident Energy (MeV)

97-Bk-241

MAT 9728

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-241



14

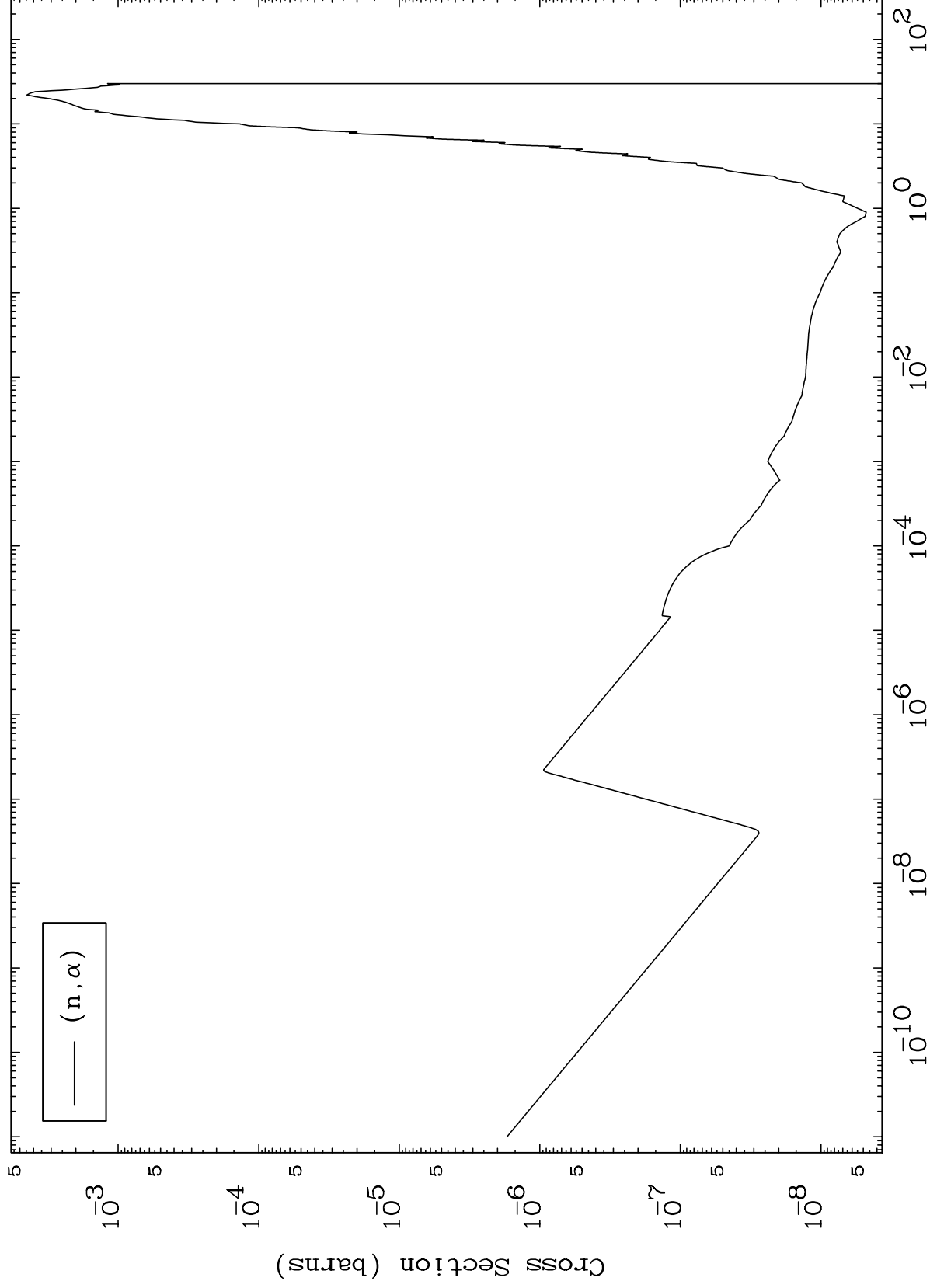
Incident Energy (MeV)

97-Bk-241

MAT 9728

(n, α) Levels
293 Kelvin Cross Sections

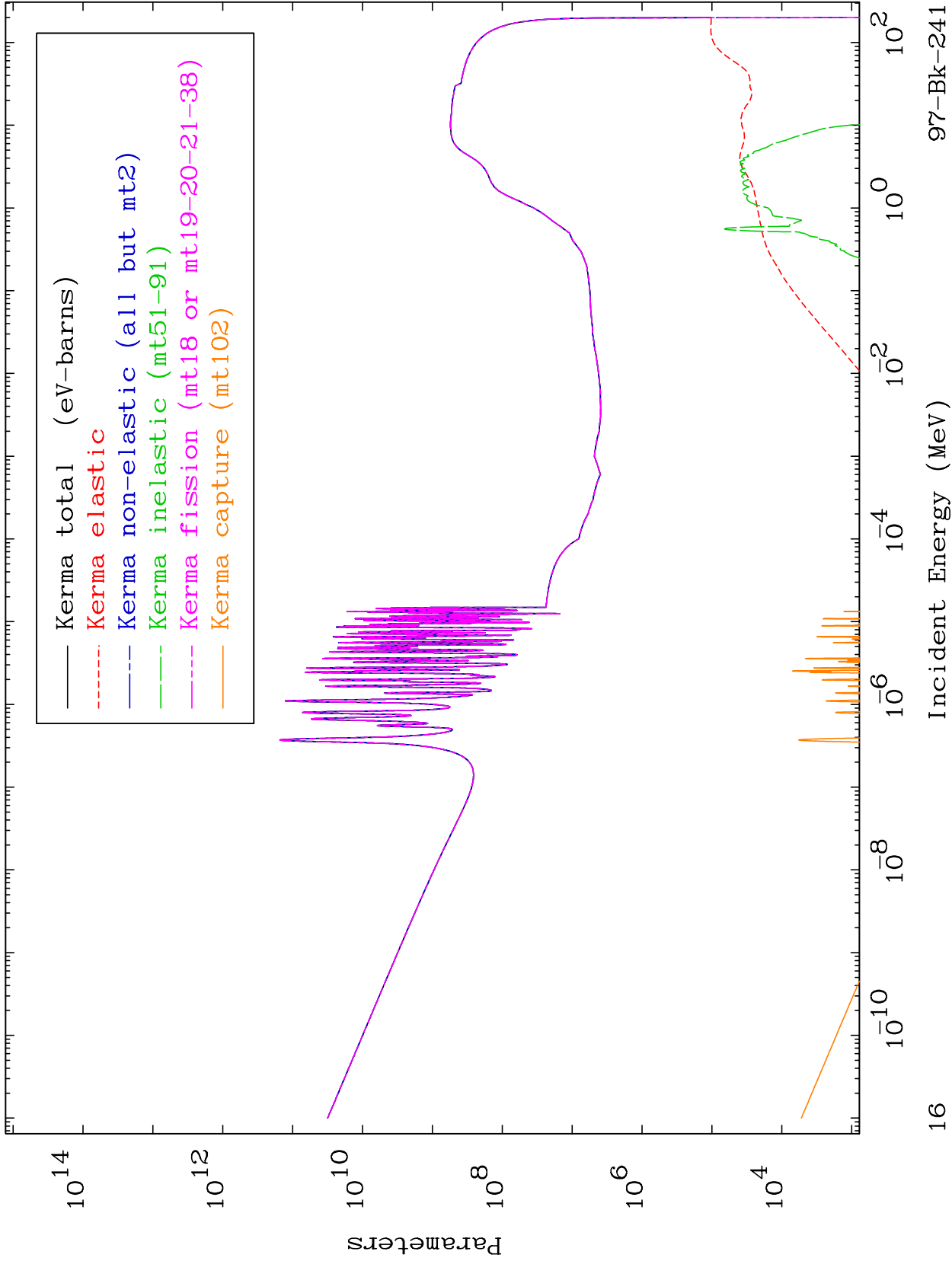
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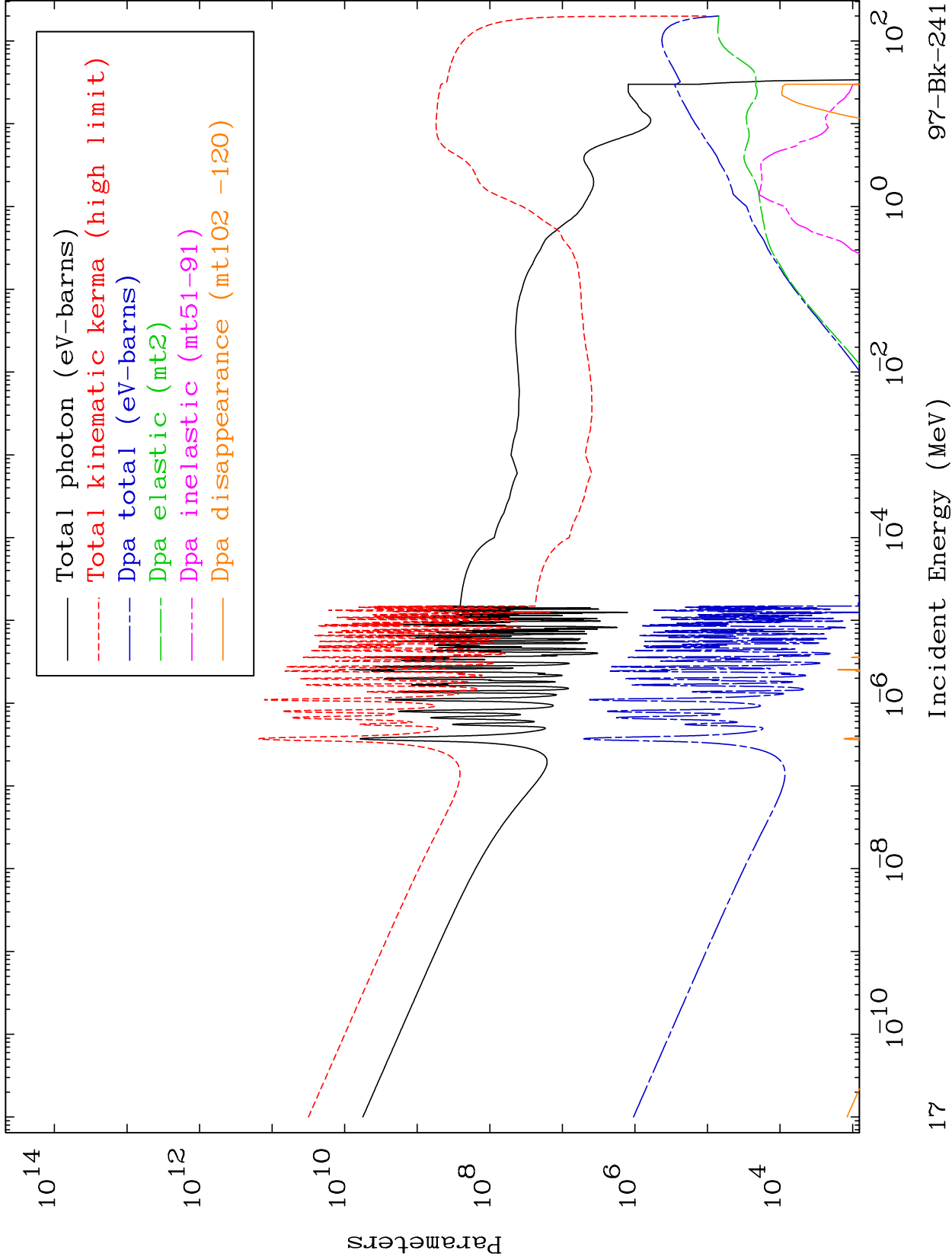


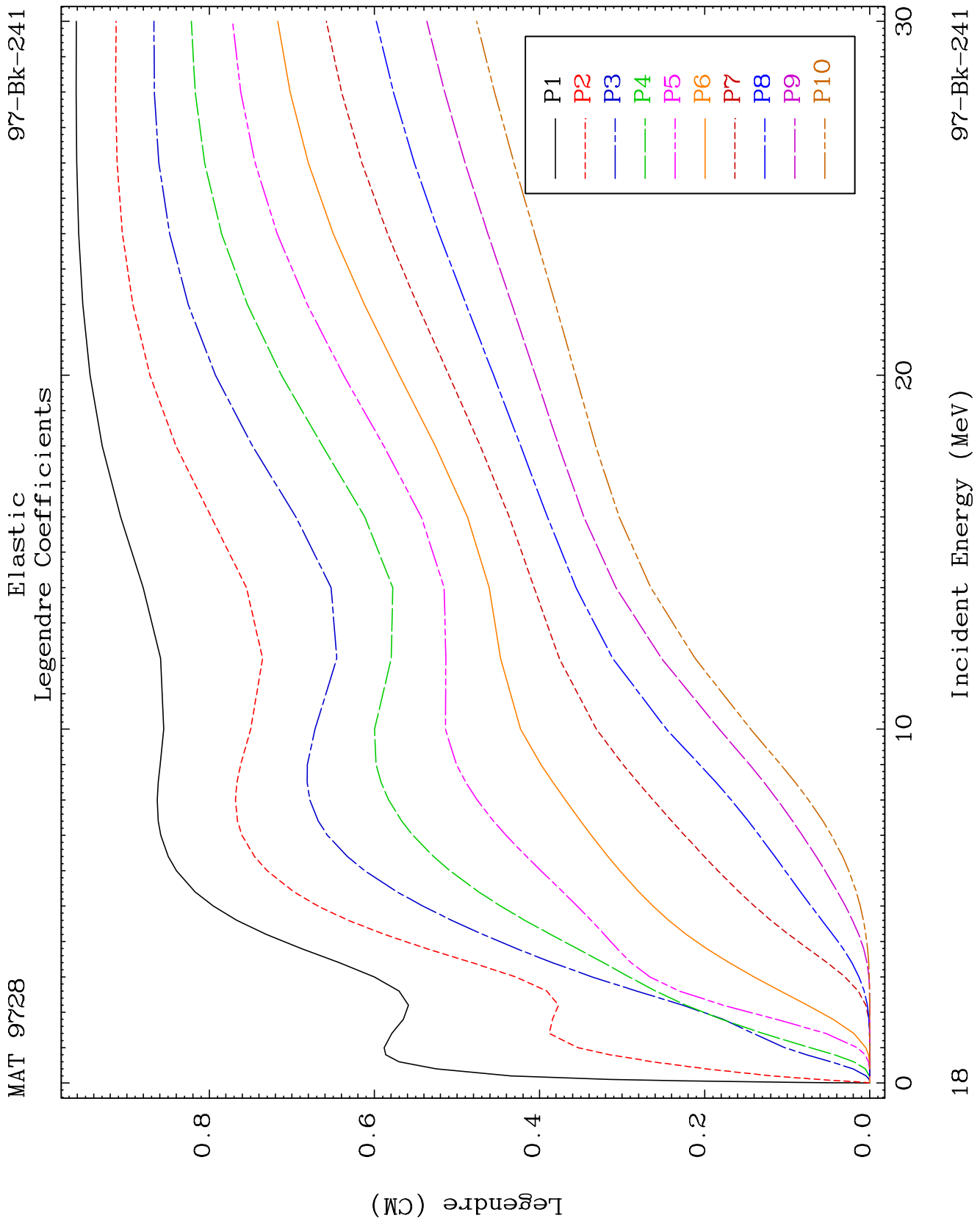
15

Incident Energy (MeV)

97-Bk-241



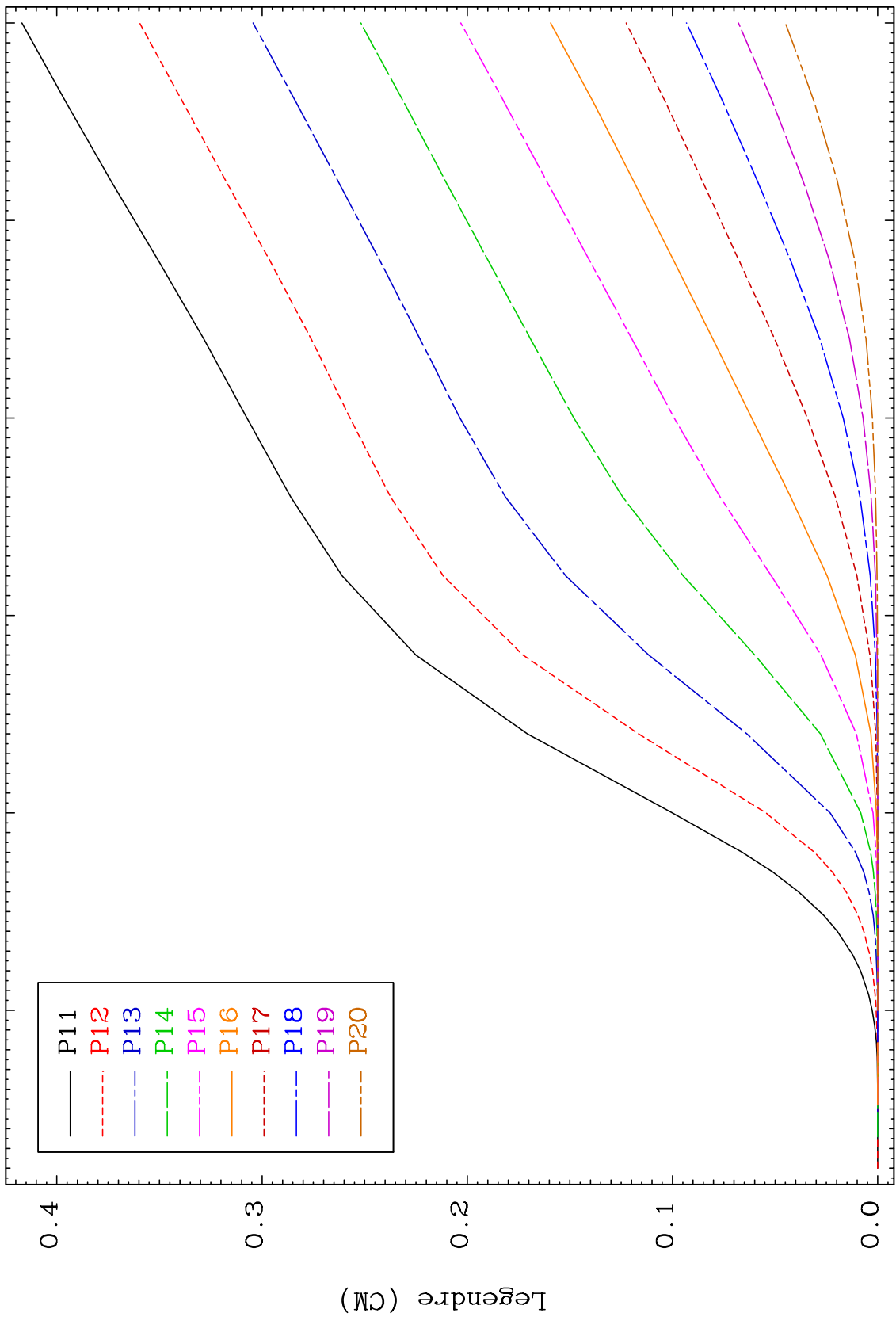




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Elastic Legendre Coefficients

97-Bk-241



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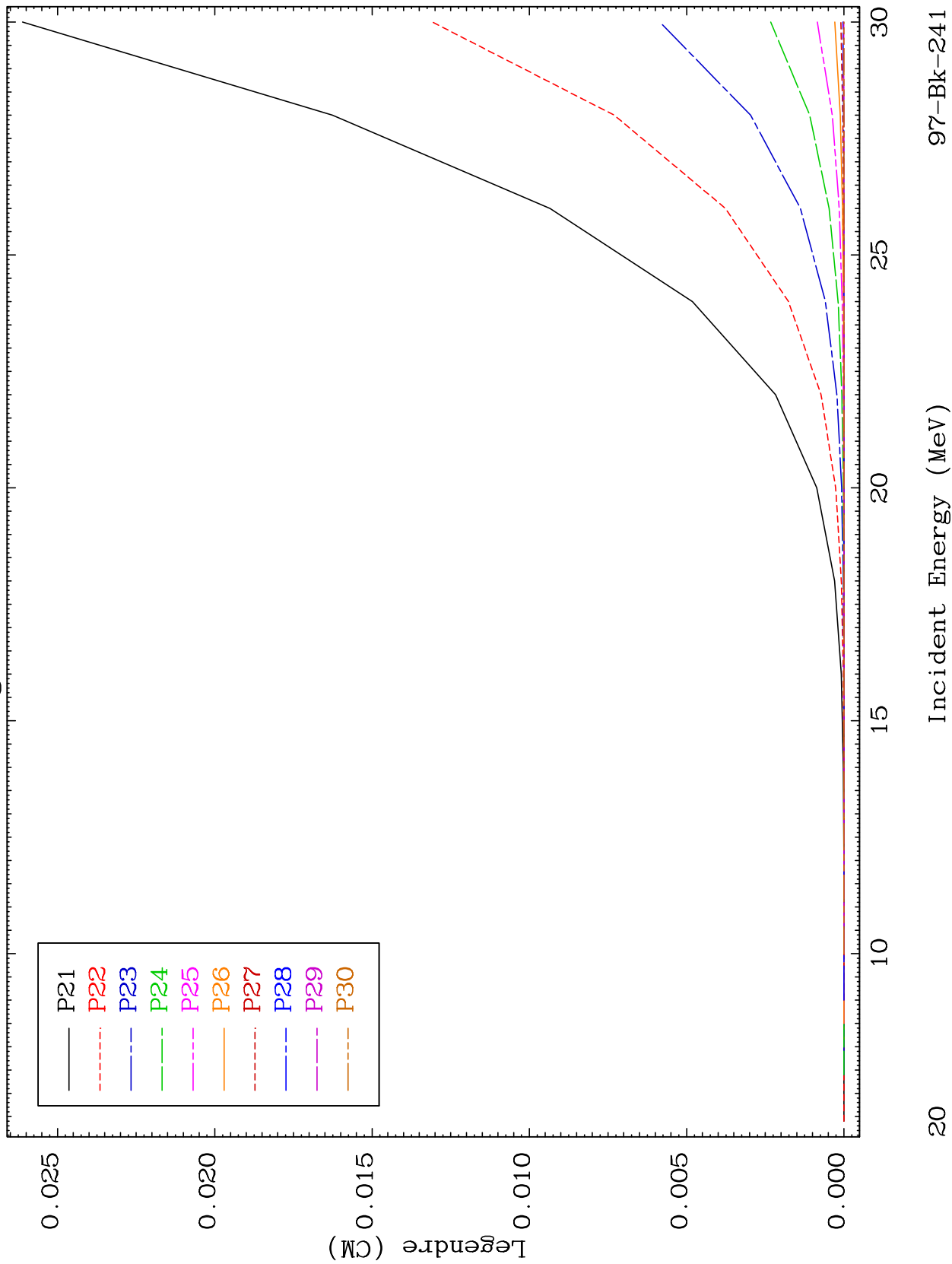
97-Bk-241

Incident Energy (MeV)

MAT 9728

Elastic Legendre Coefficients

97-Bk-241



97-Bk-241

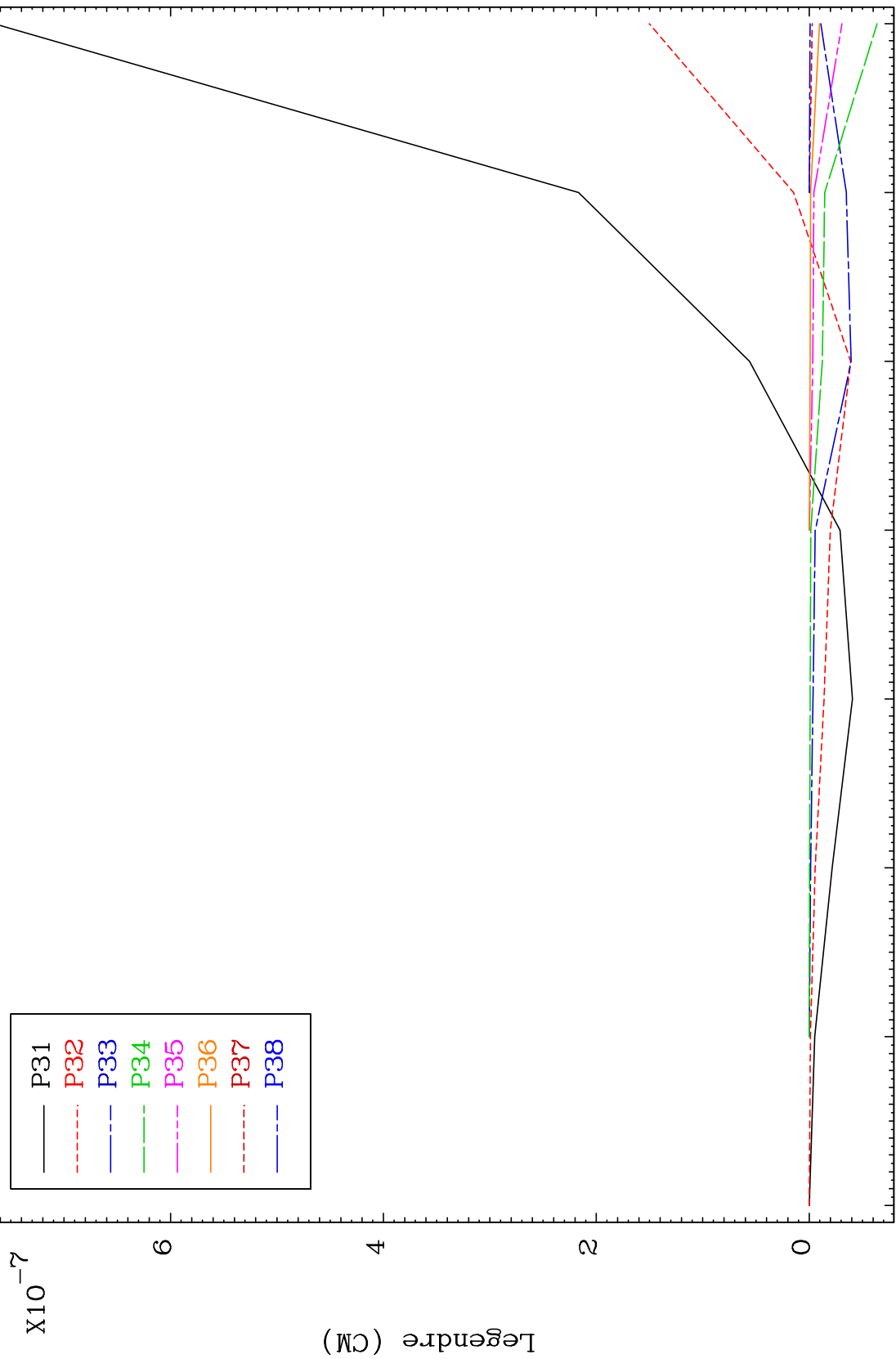
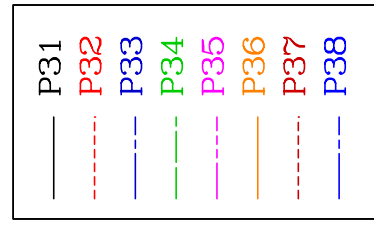
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

97-Bk-241



21

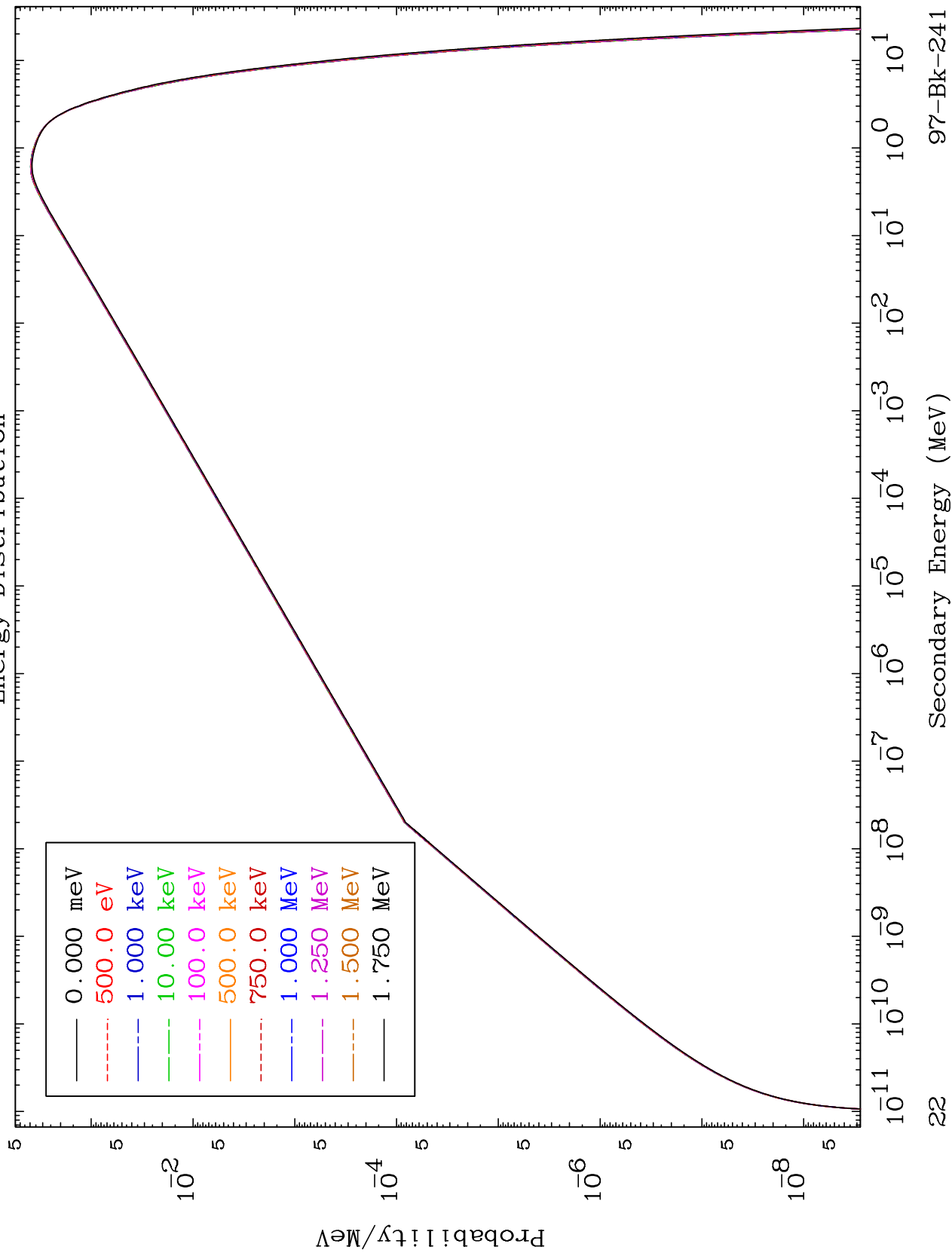
Incident Energy (MeV)

97-Bk-241

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Fission Energy Distribution

97-Bk-241



97-Bk-241

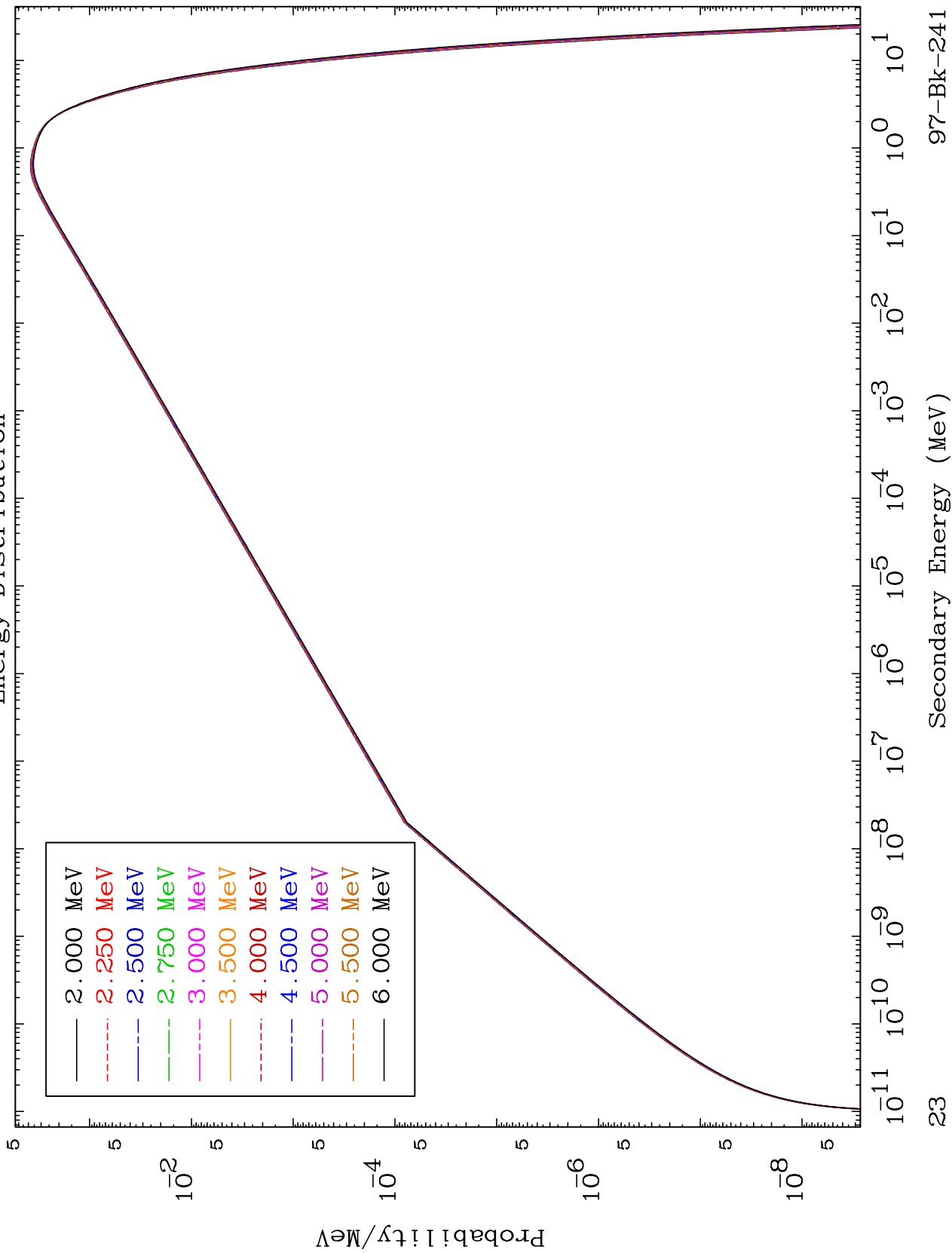
Secondary Energy (MeV)

22

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Fission Energy Distribution

97-Bk-241



97-Bk-241

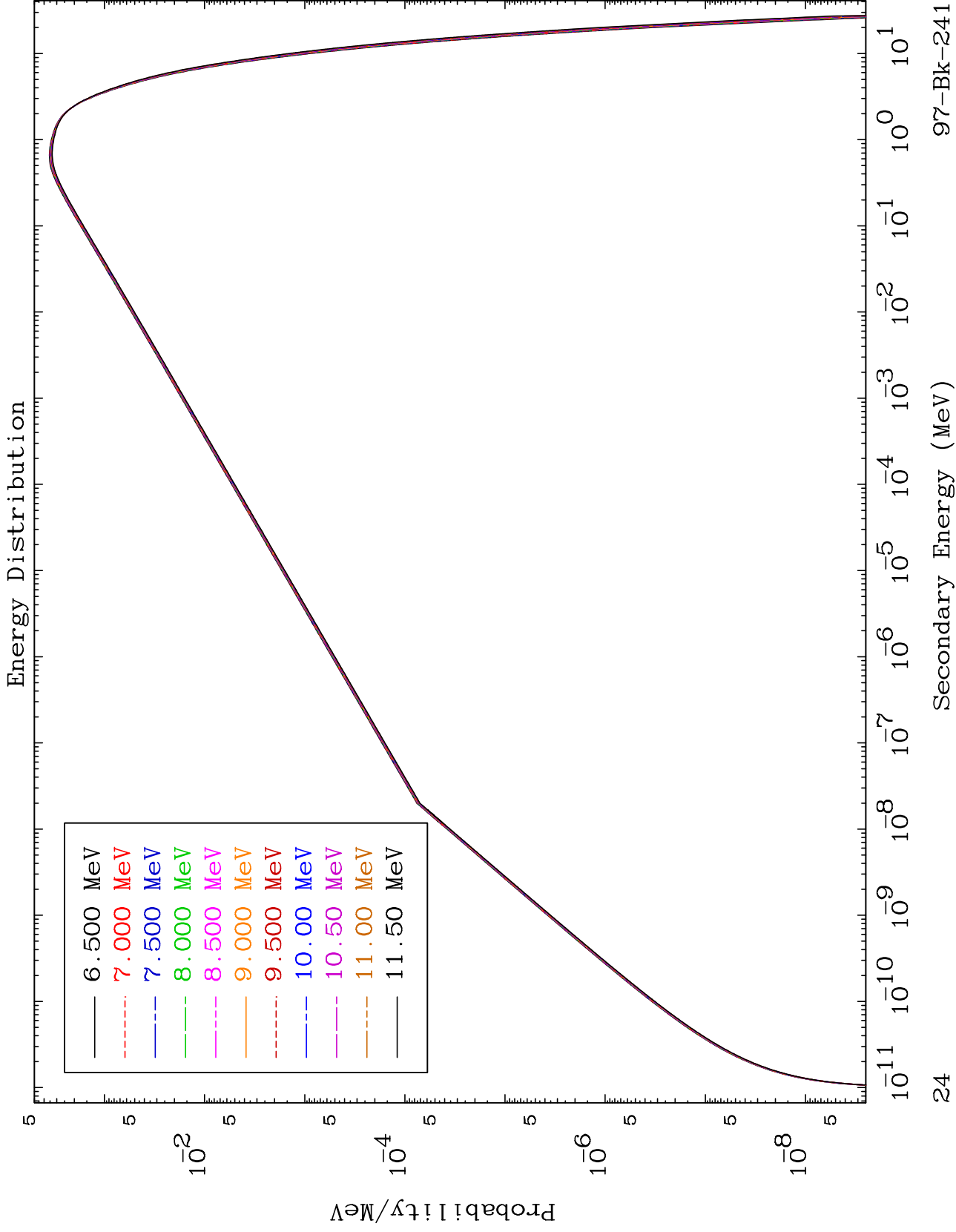
Secondary Energy (MeV)

23

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Fission

97-Bk-241

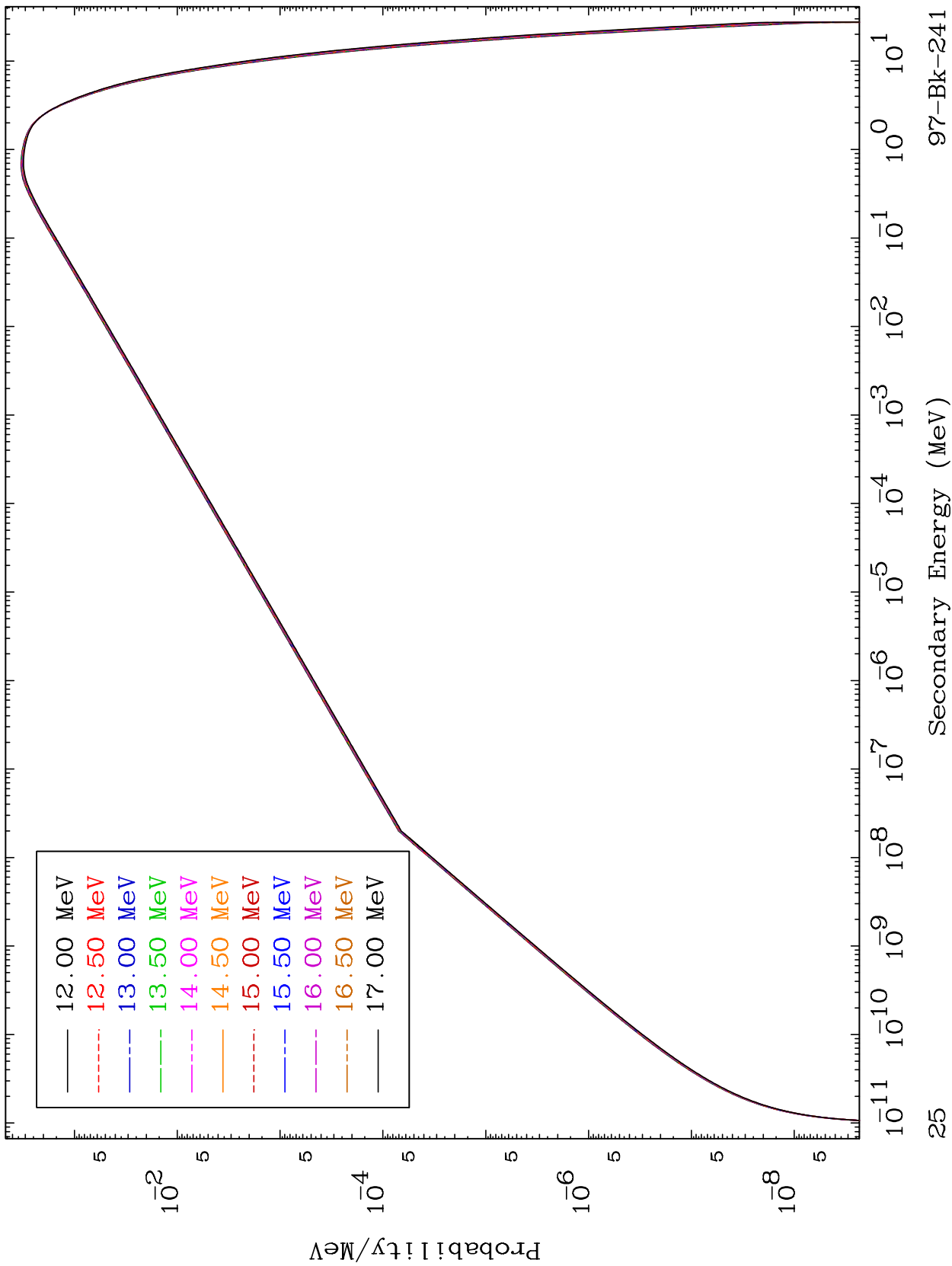


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Fission Energy Distribution

97-Bk-241



97-Bk-241

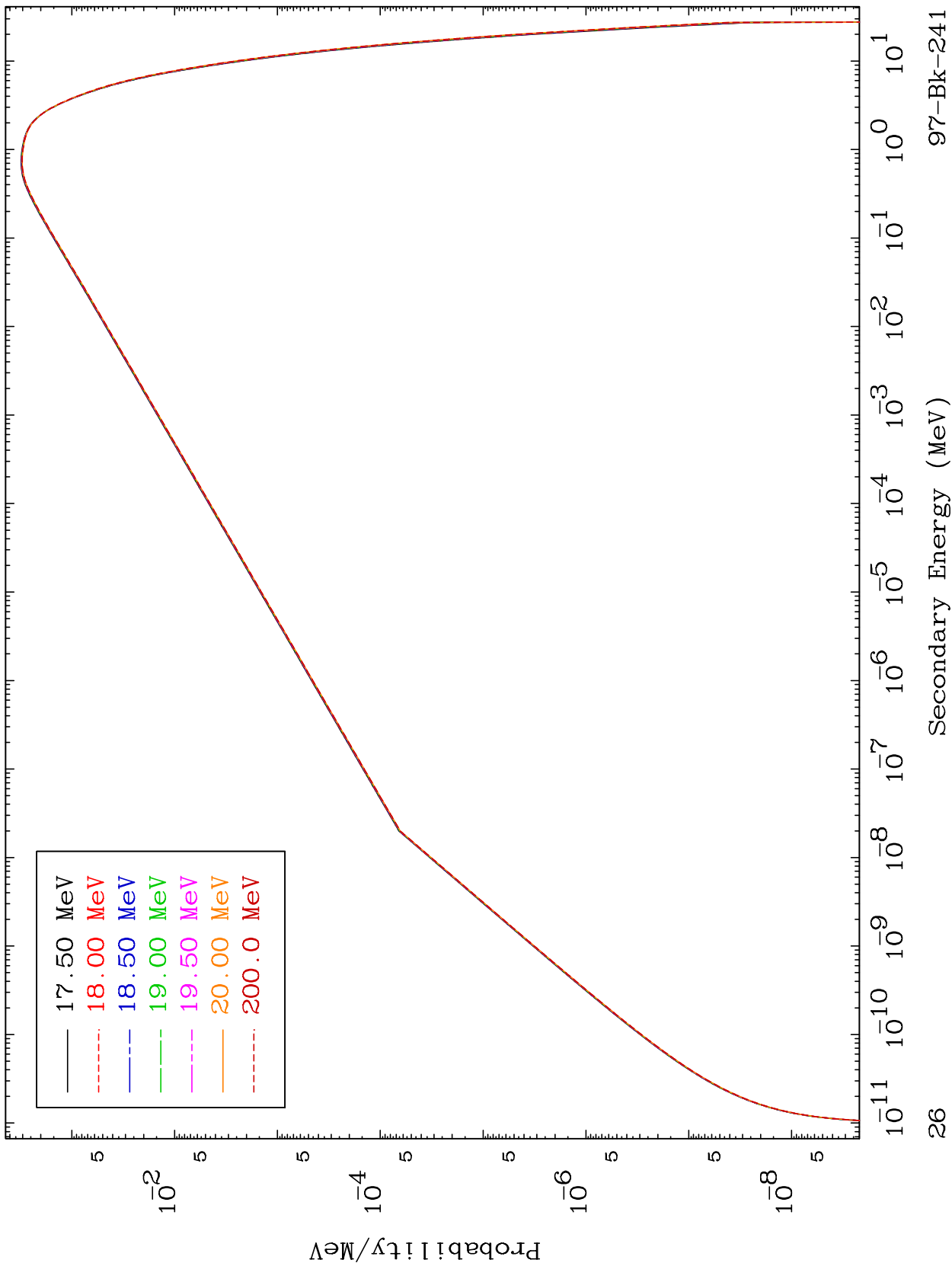
Secondary Energy (MeV)

25

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Fission Energy Distribution

97-Bk-241

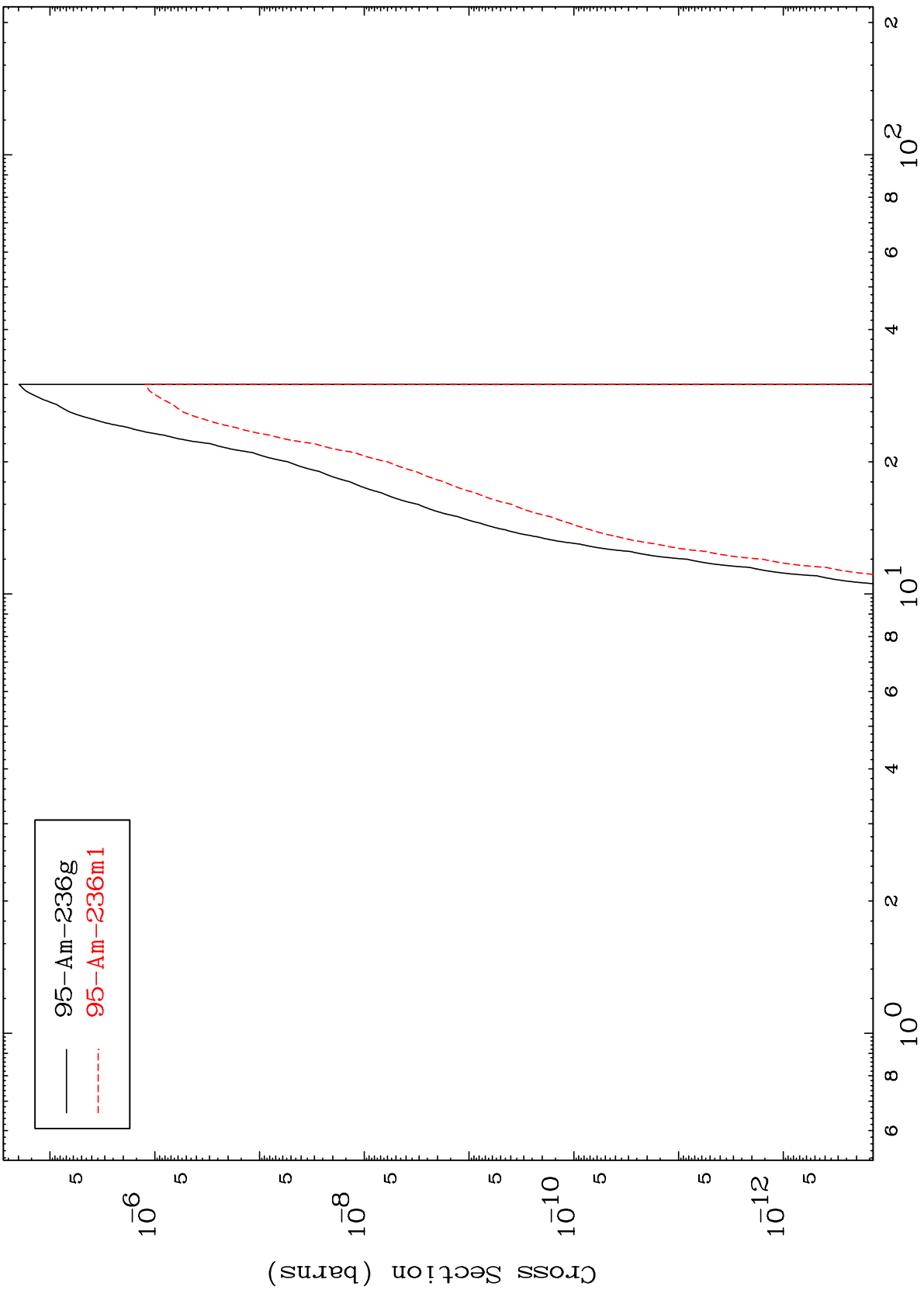


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(n,2n) α

97-Bk-241

Radionuclide Production Cross Section



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Incident Energy (MeV)

97-Bk-241

MAT 9728

(n,p) α

97-Bk-241

Radionuclide Production Cross Section

