

Program EVALPLOT
(Version 2018-1)

by

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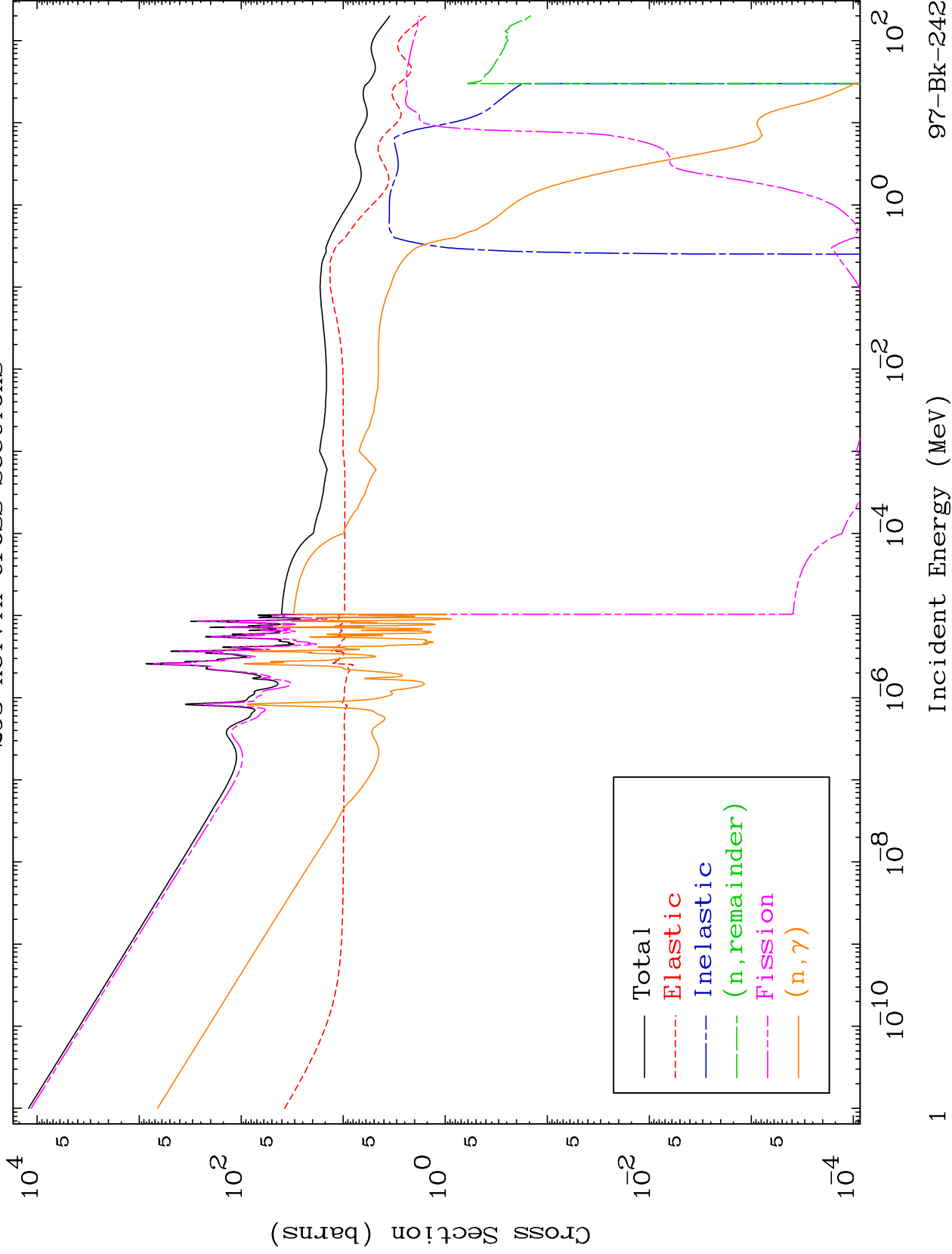
Press Mouse Button to Start

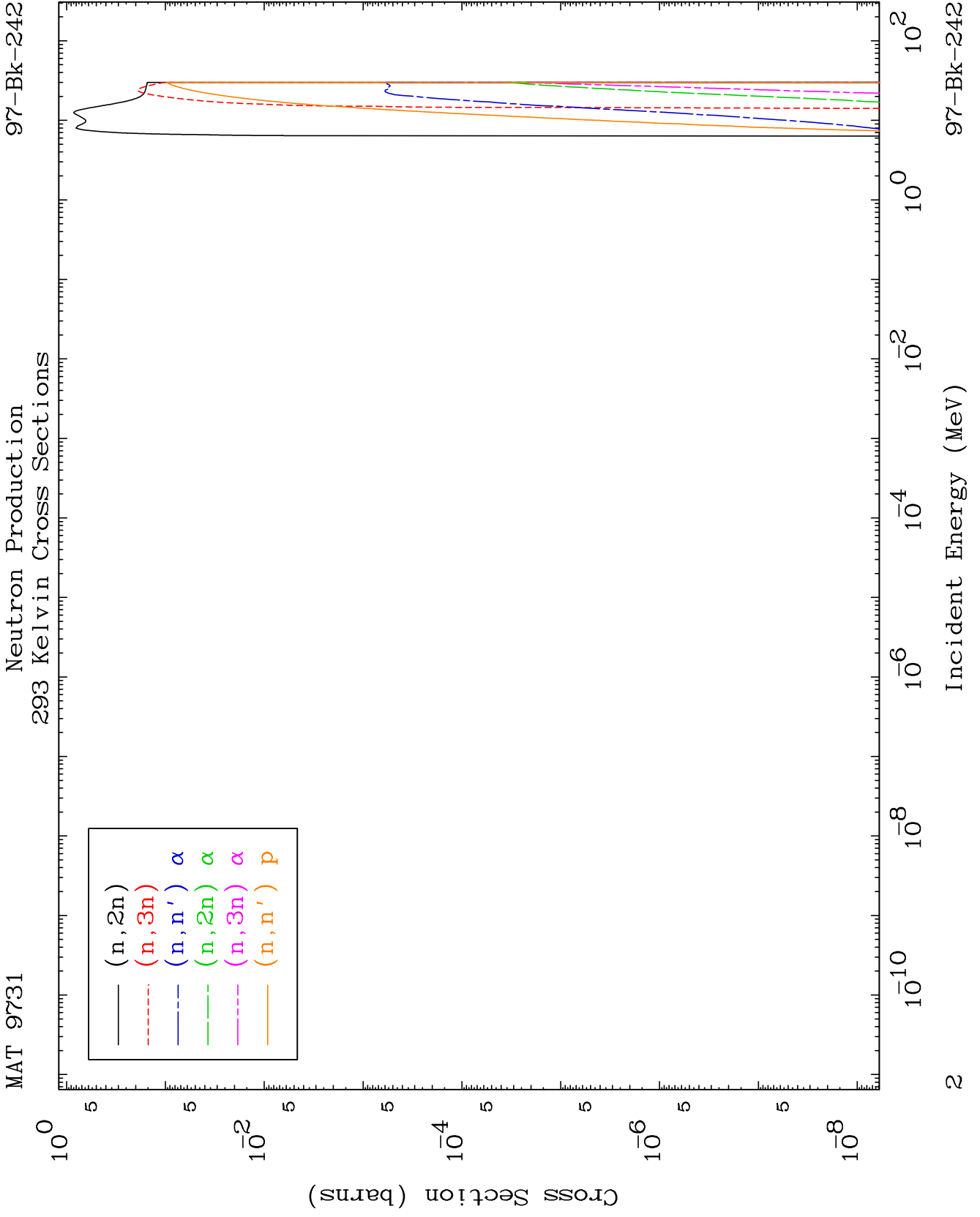
MAT 9731

Major

293 Kelvin Cross Sections

97-Bk-242

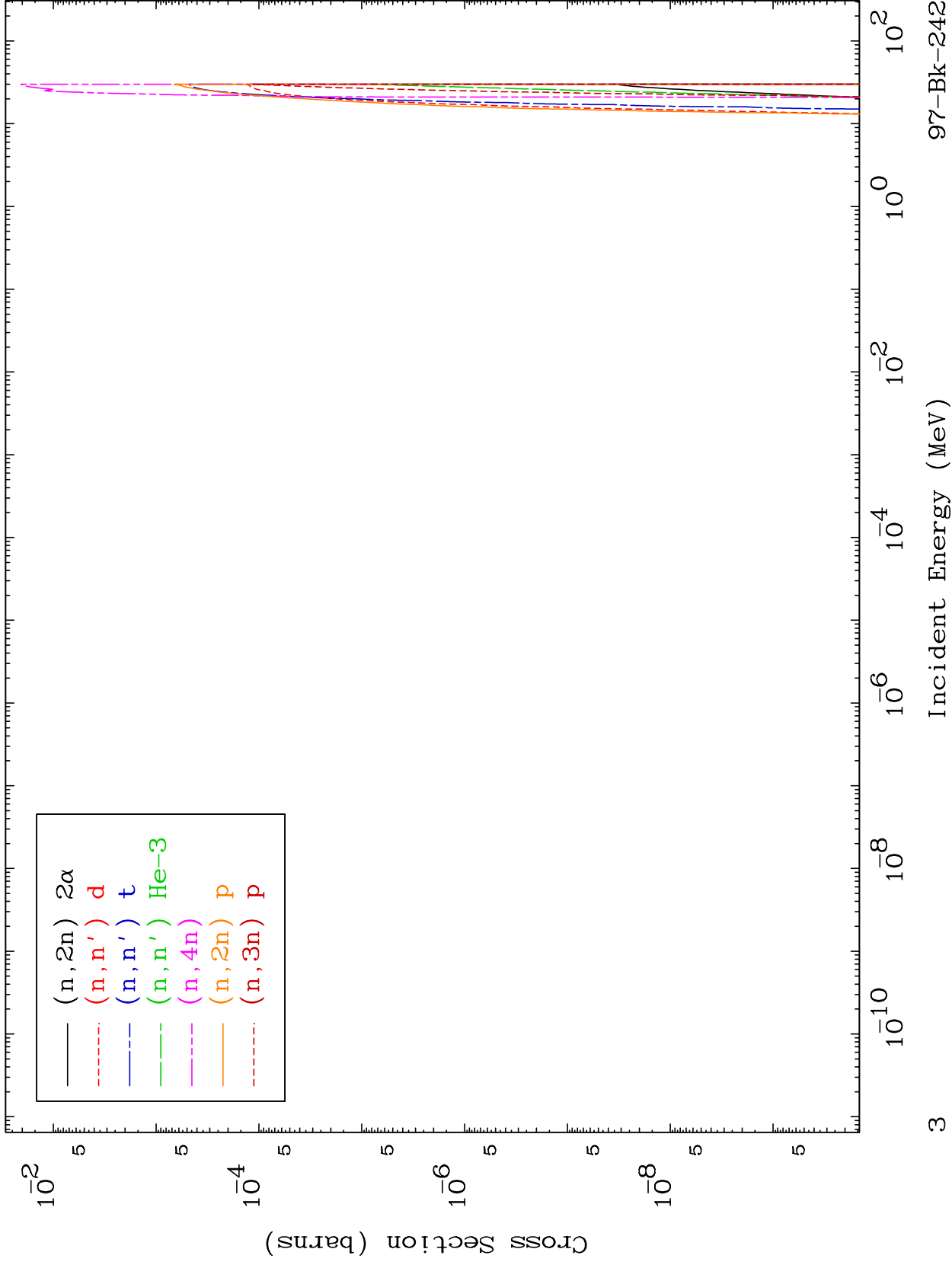




MAT 9731

Neutron Production
293 Kelvin Cross Sections

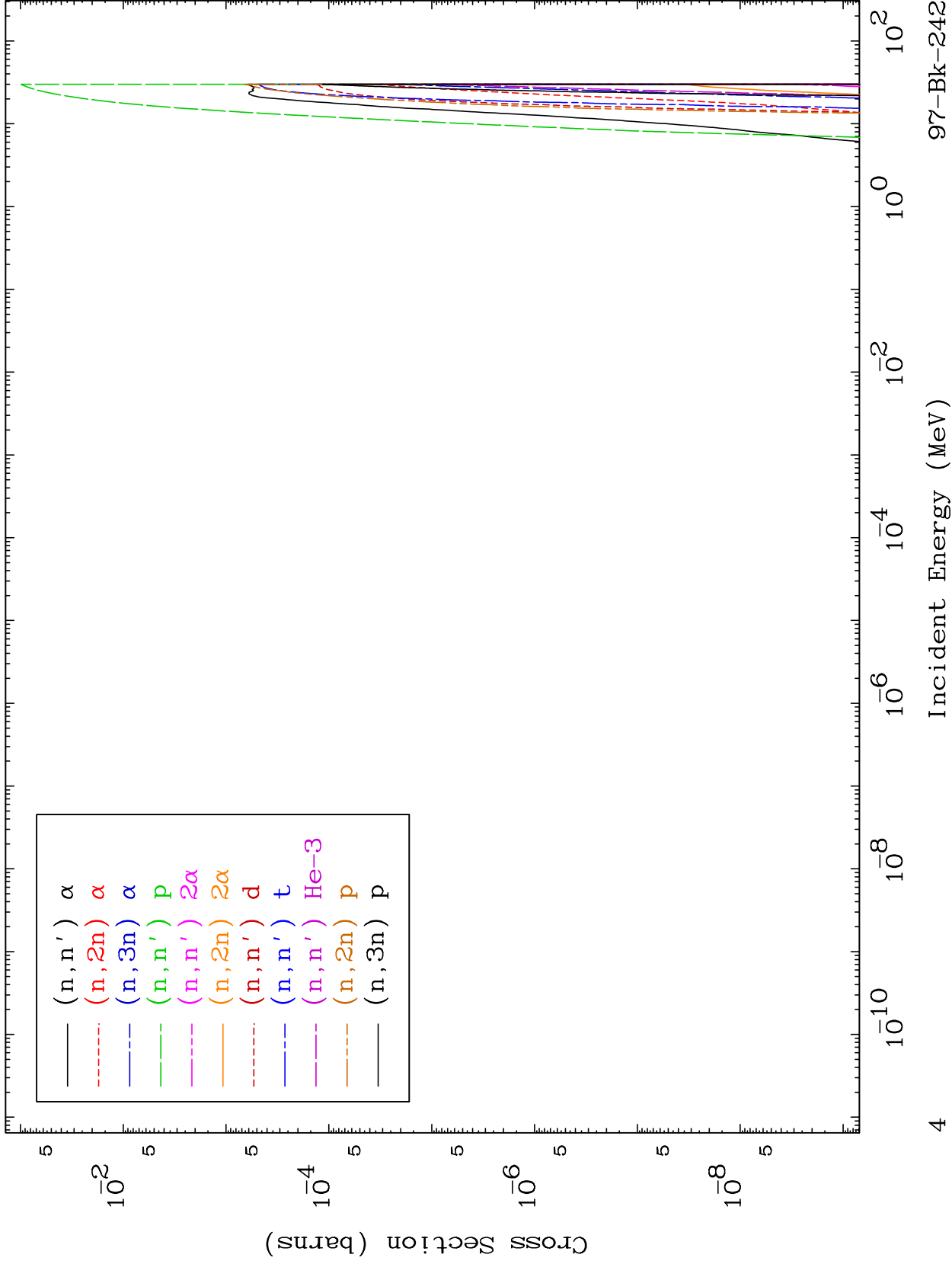
97-Bk-242



MAT 9731

Charged Particle
293 Kelvin Cross Sections

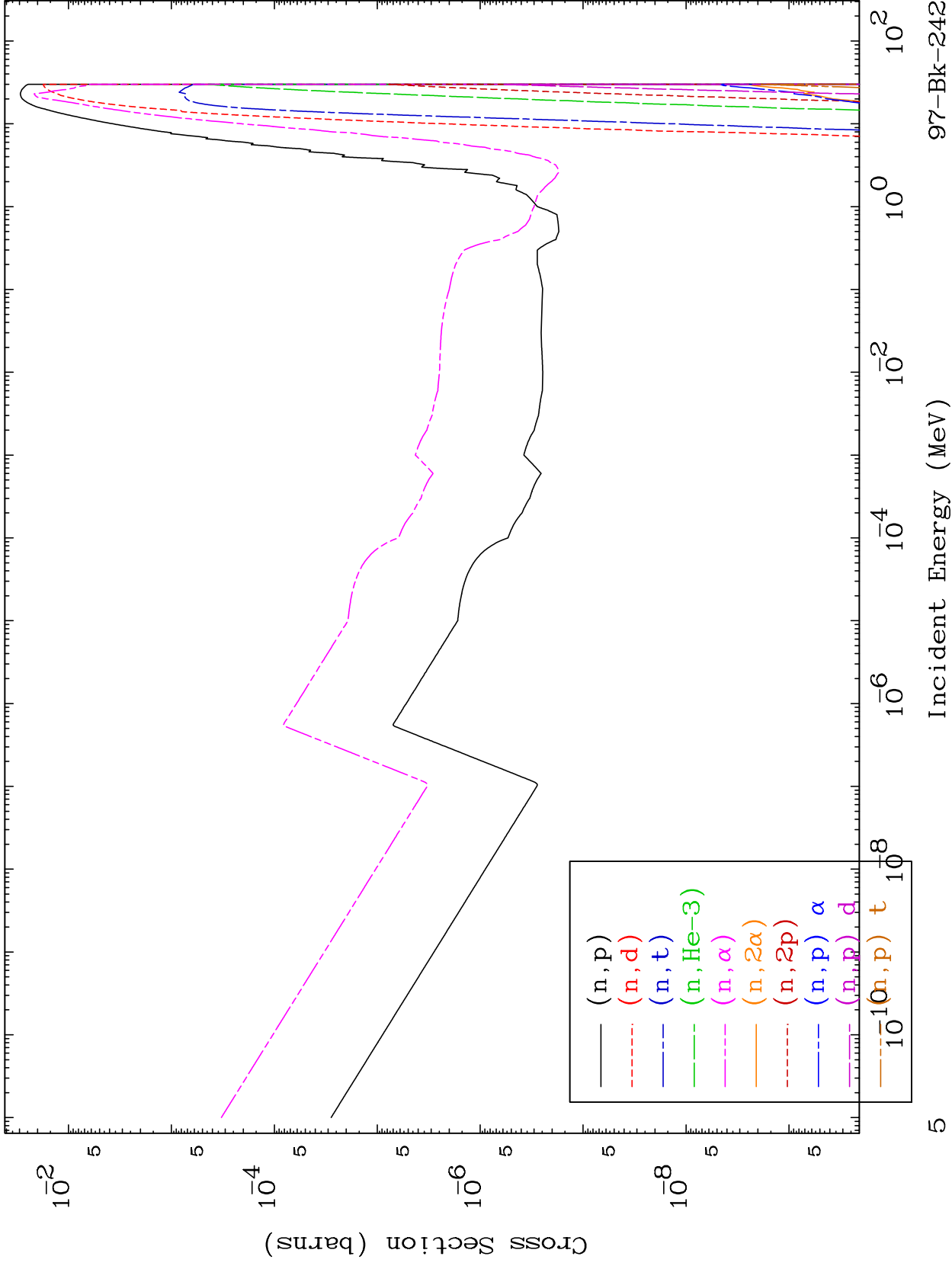
97-Bk-242



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Charged Particle
293 Kelvin Cross Sections

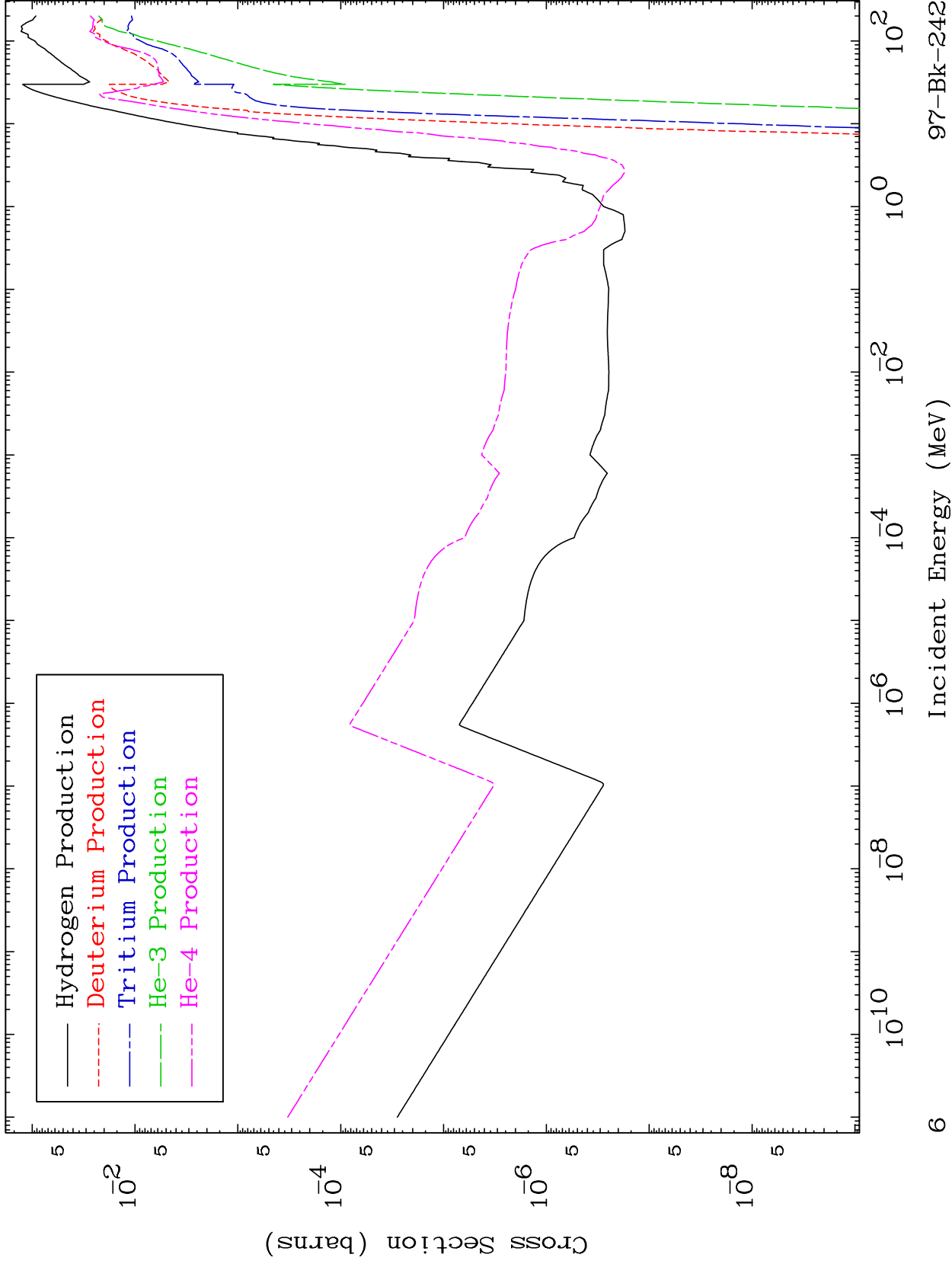
97-Bk-242



MAT 9731

Particle Production
293 Kelvin Cross Sections

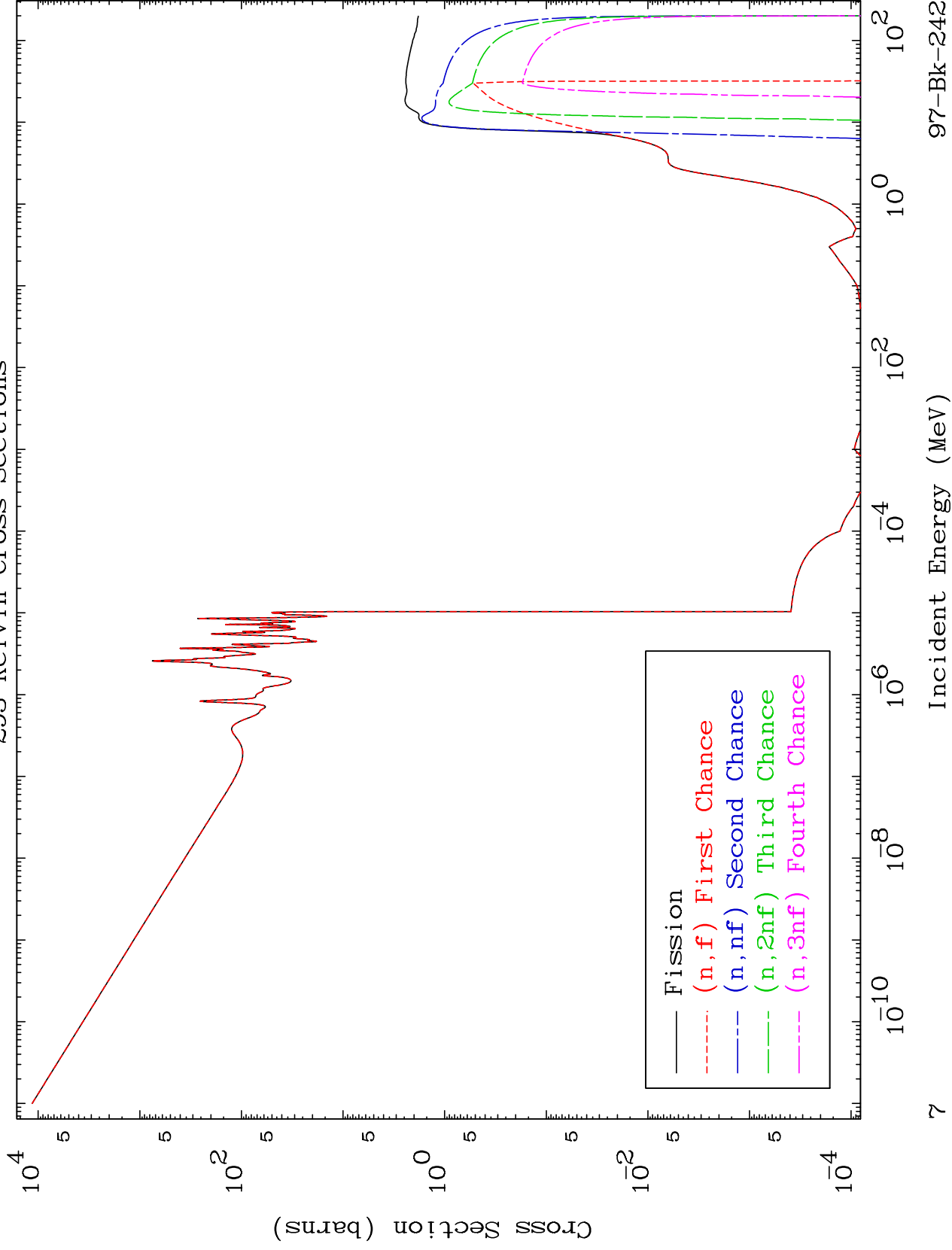
97-Bk-242



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Fission
293 Kelvin Cross Sections

97-Bk-242

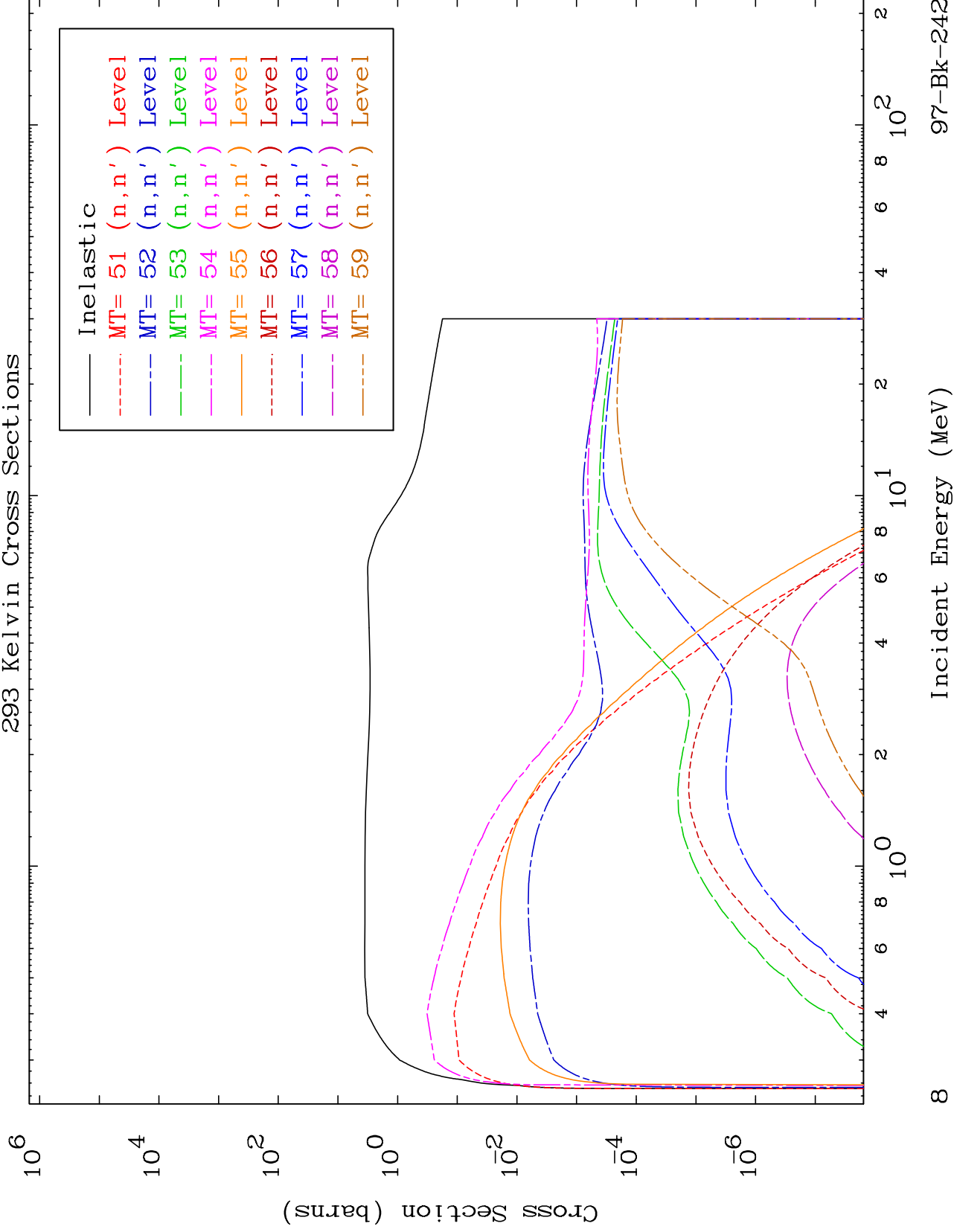


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(n,n') Level

97-Bk-242

293 Kelvin Cross Sections

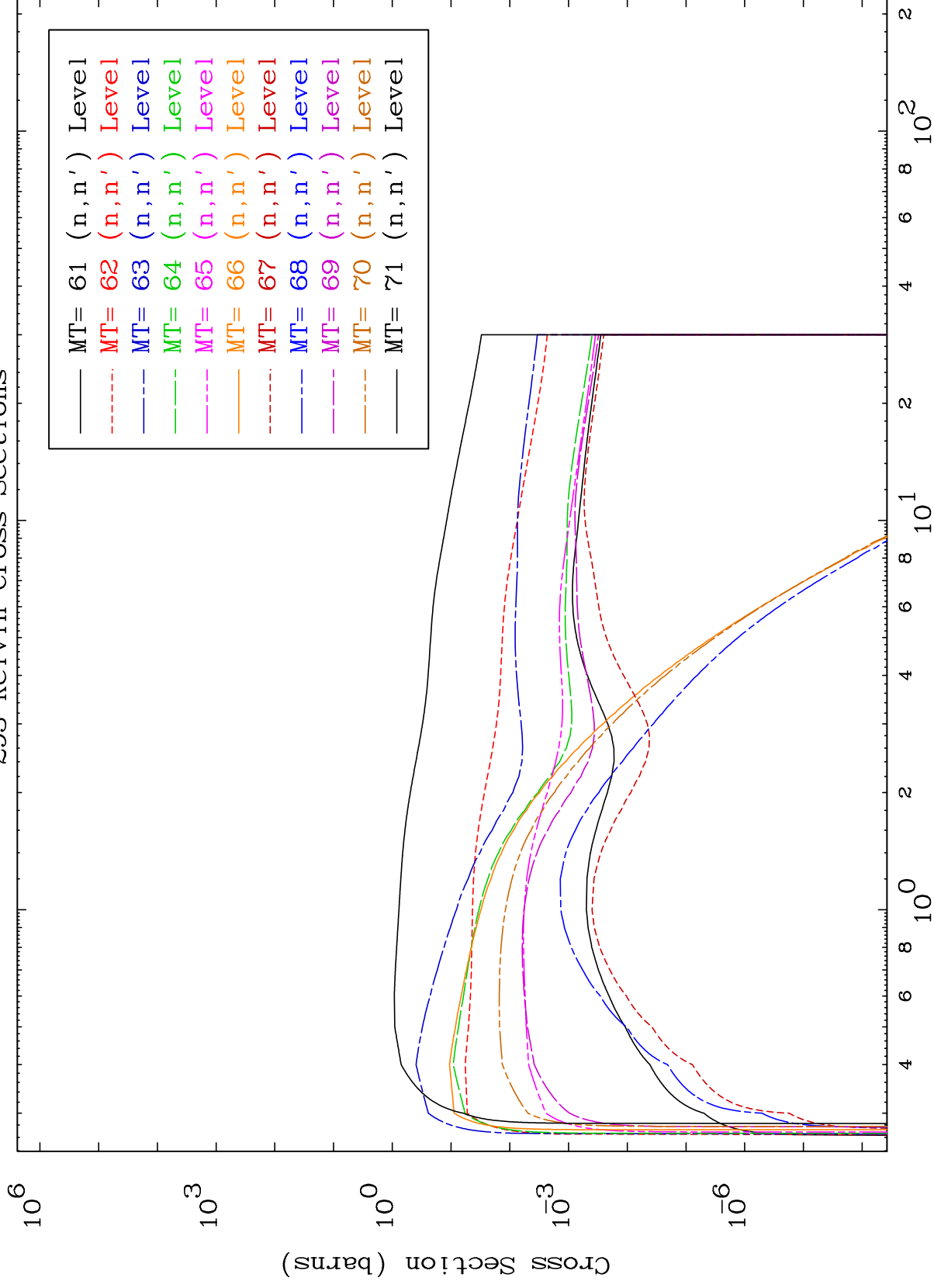


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(n,n') Level

97-Bk-242

293 Kelvin Cross Sections

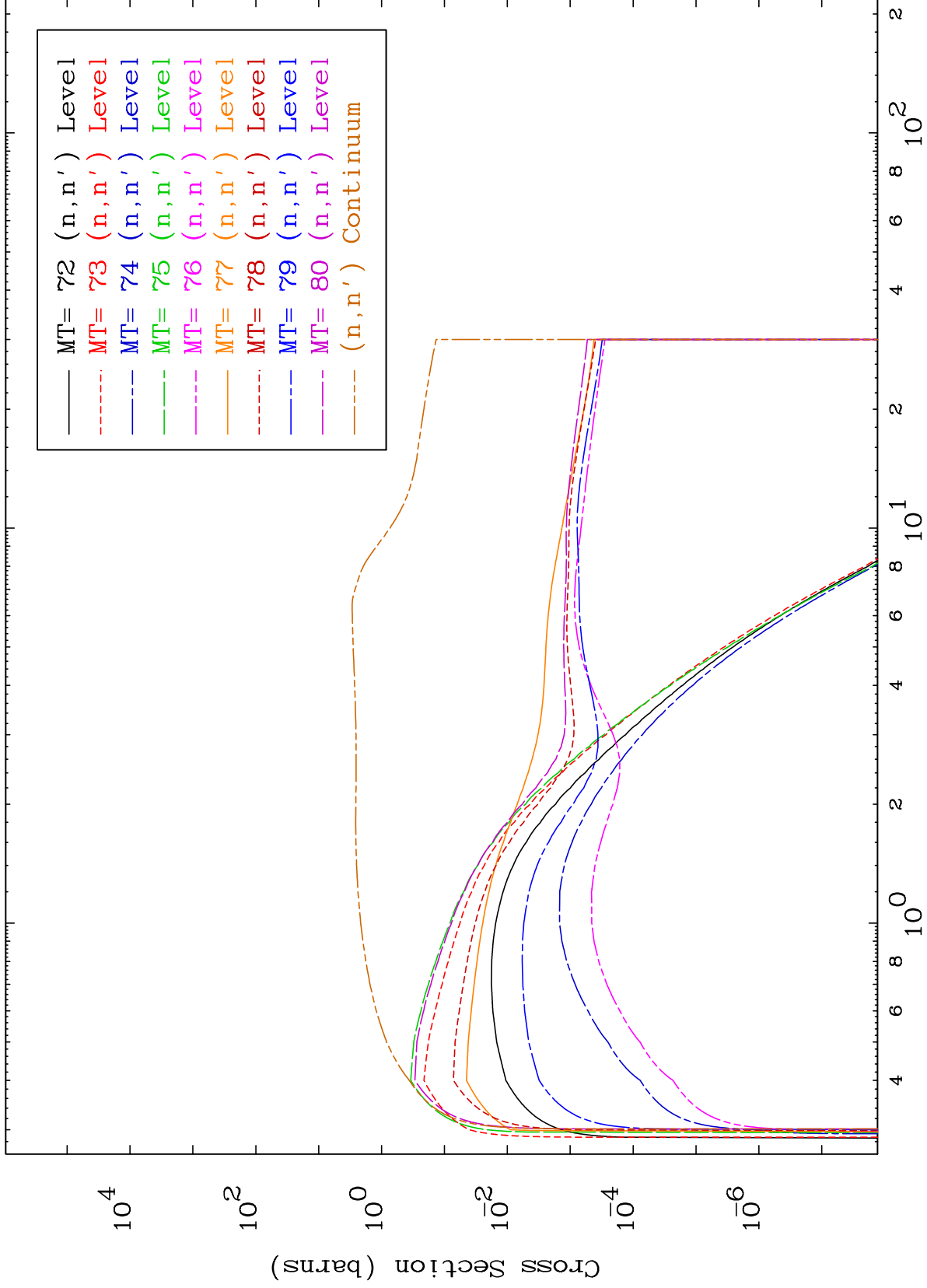


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(n,n') Level

97-Bk-242

293 Kelvin Cross Sections



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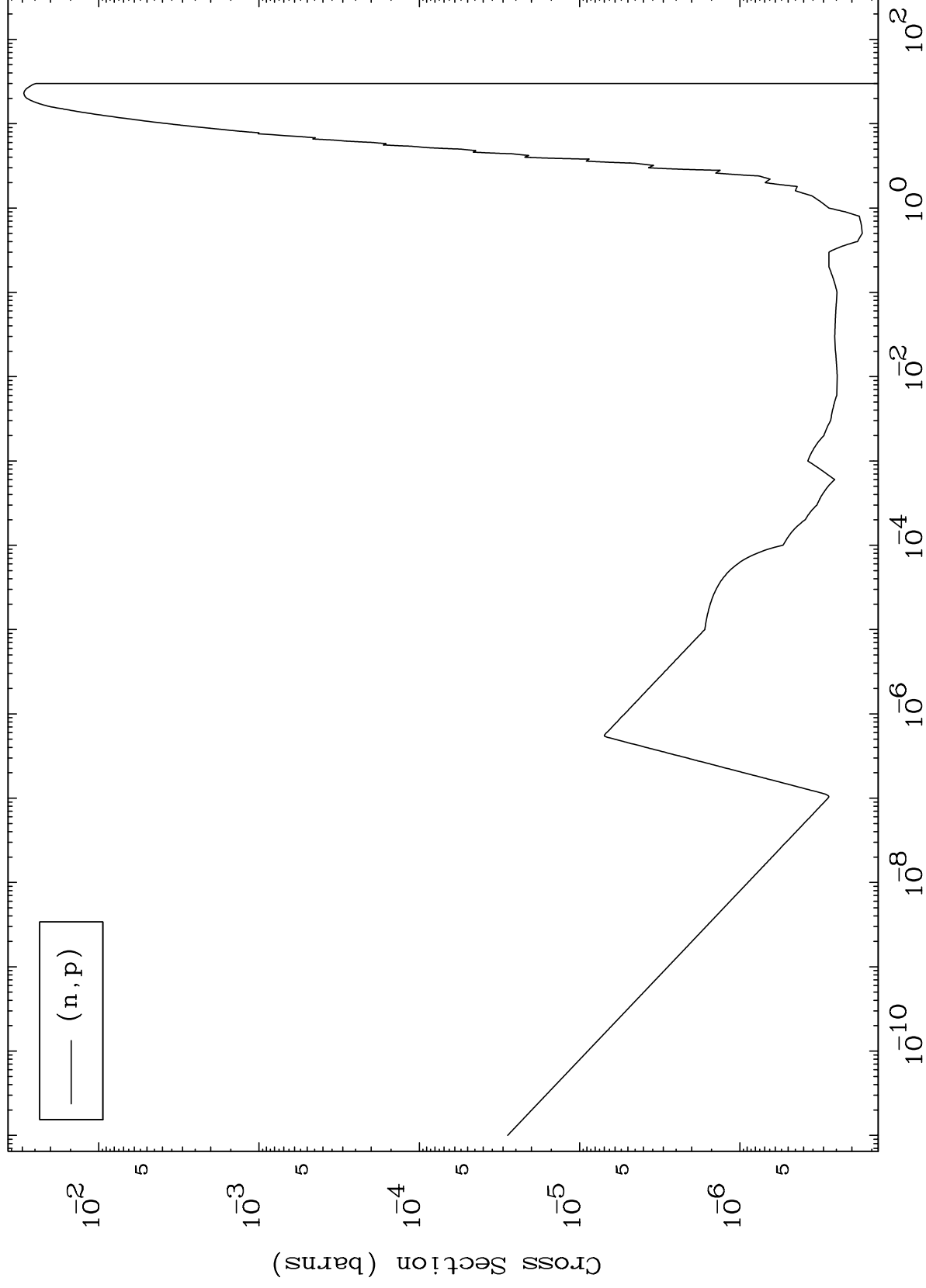
Incident Energy (MeV)

97-Bk-242

MAT 9731

(n,p) Levels
293 Kelvin Cross Sections

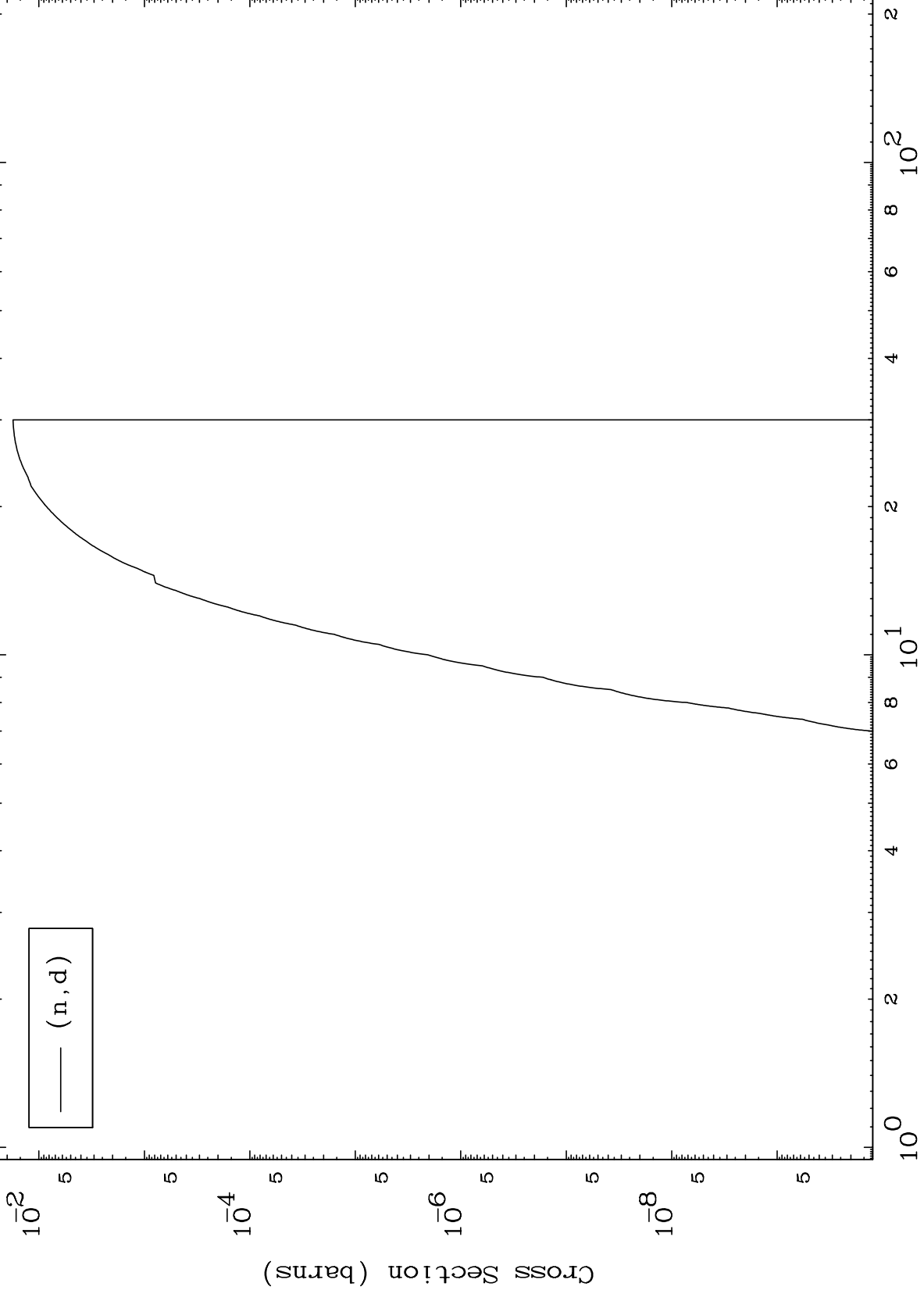
97-Bk-242



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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-242



12

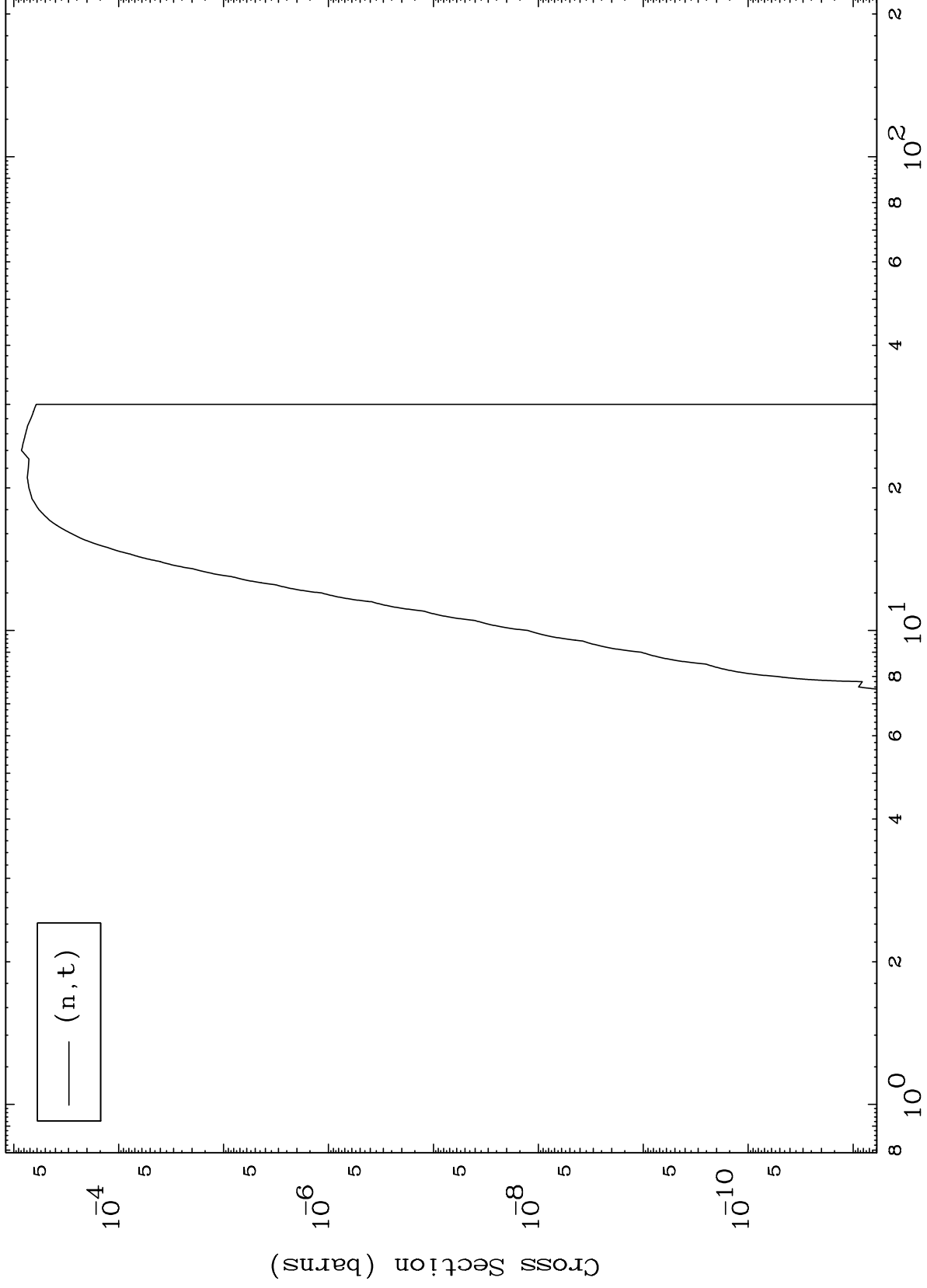
Incident Energy (MeV)

97-Bk-242

MAT 9731

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-242



13

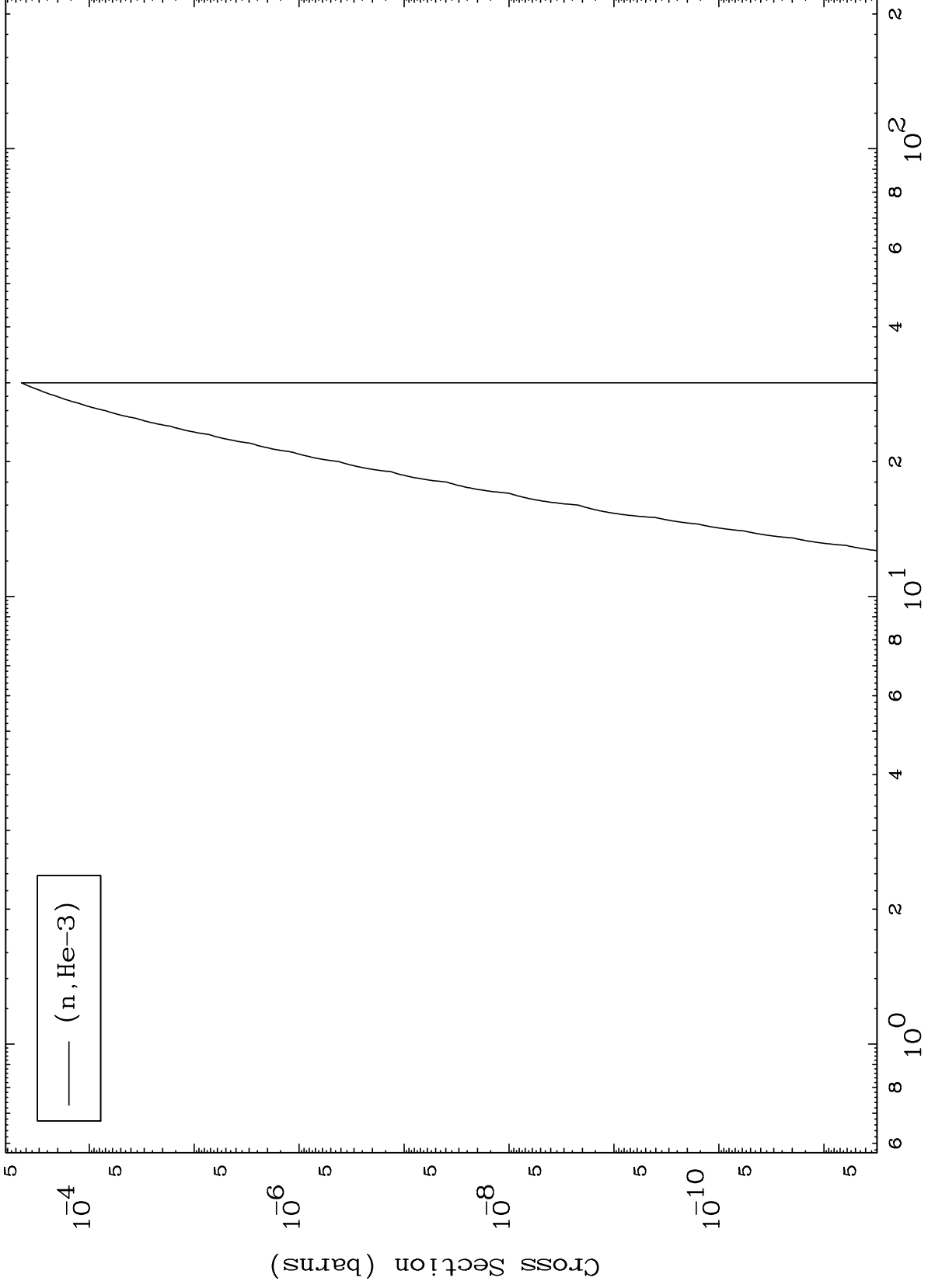
Incident Energy (MeV)

97-Bk-242

MAT 9731

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-242



14

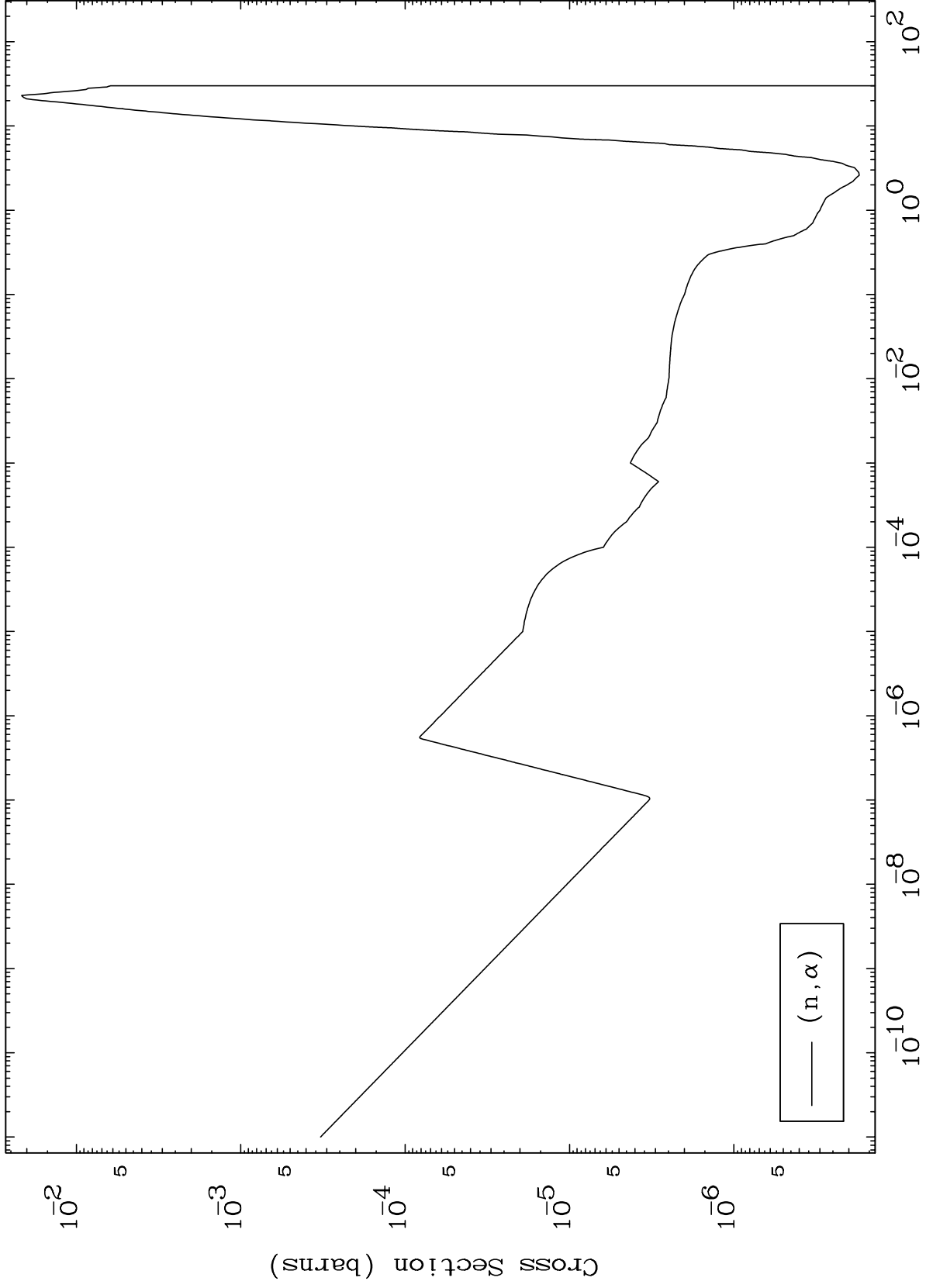
Incident Energy (MeV)

97-Bk-242

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(n,α) Levels
293 Kelvin Cross Sections

97-Bk-242



15

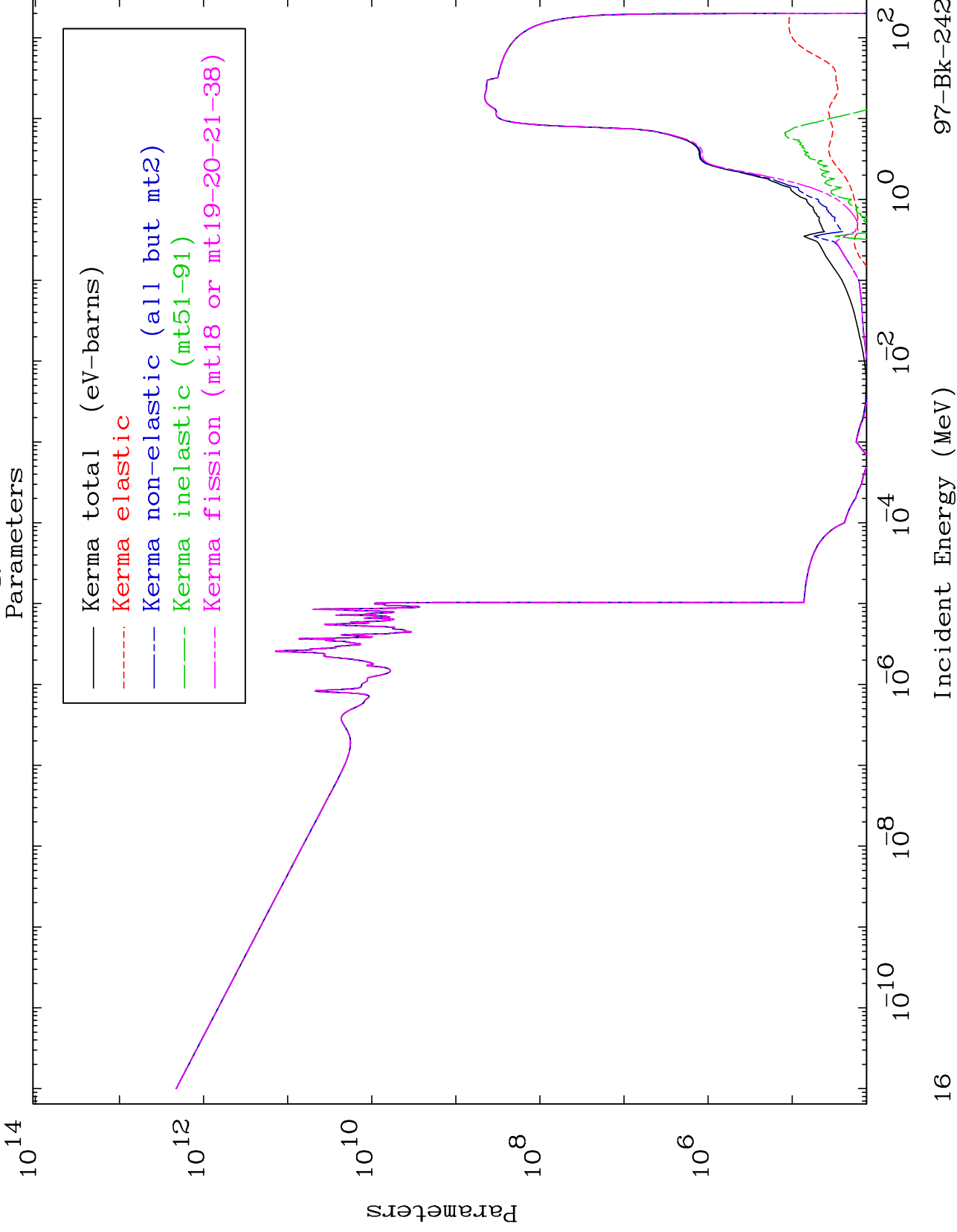
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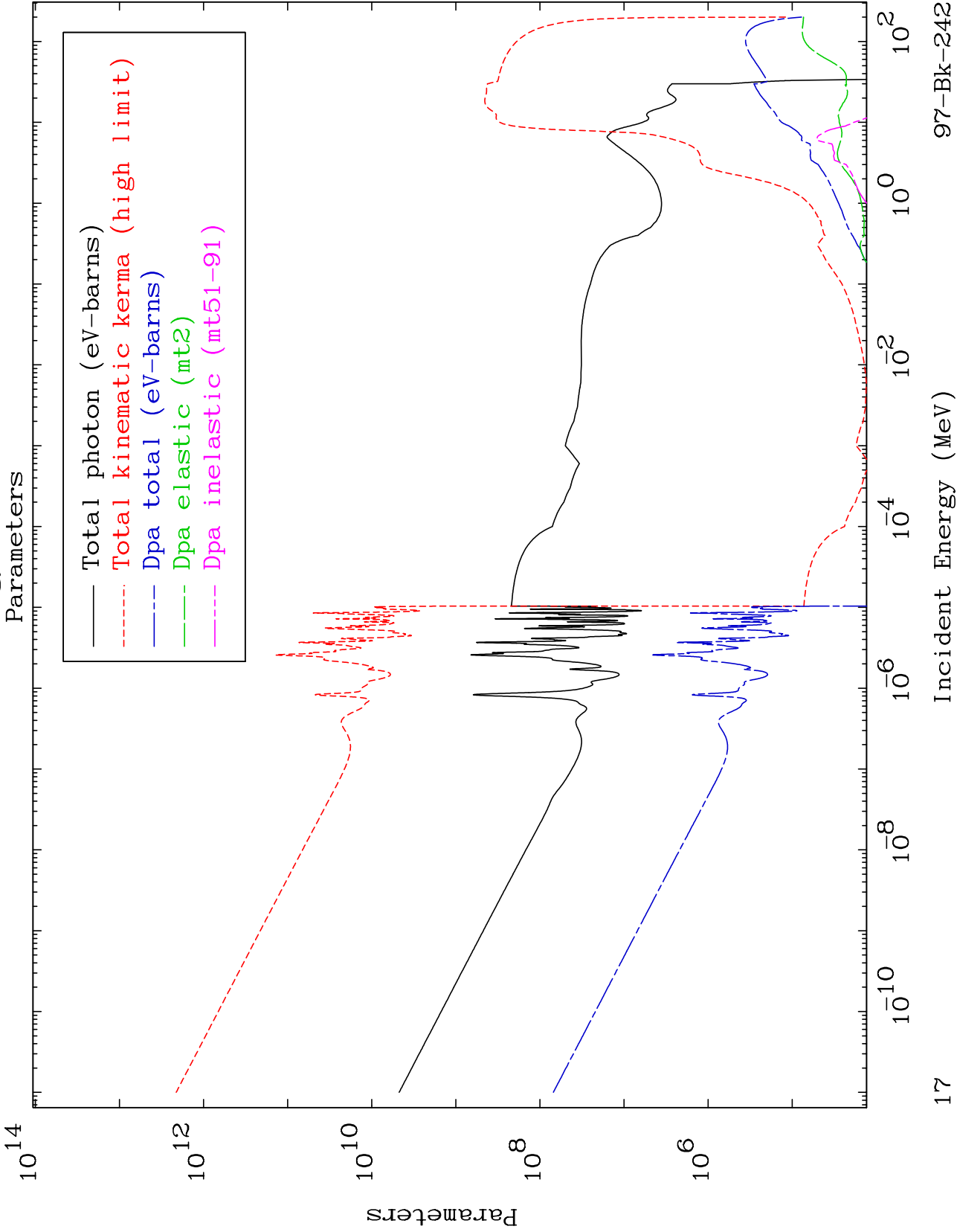
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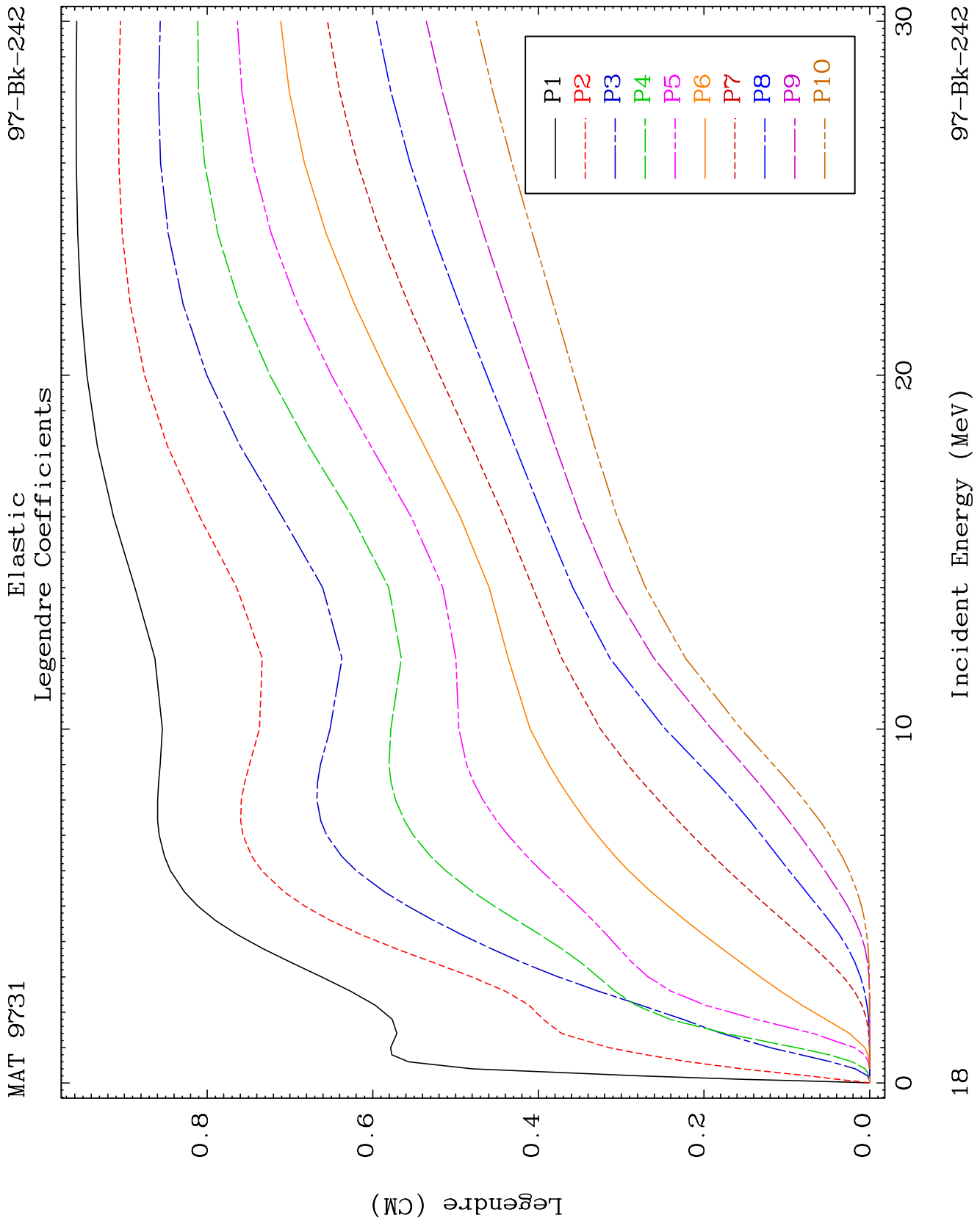
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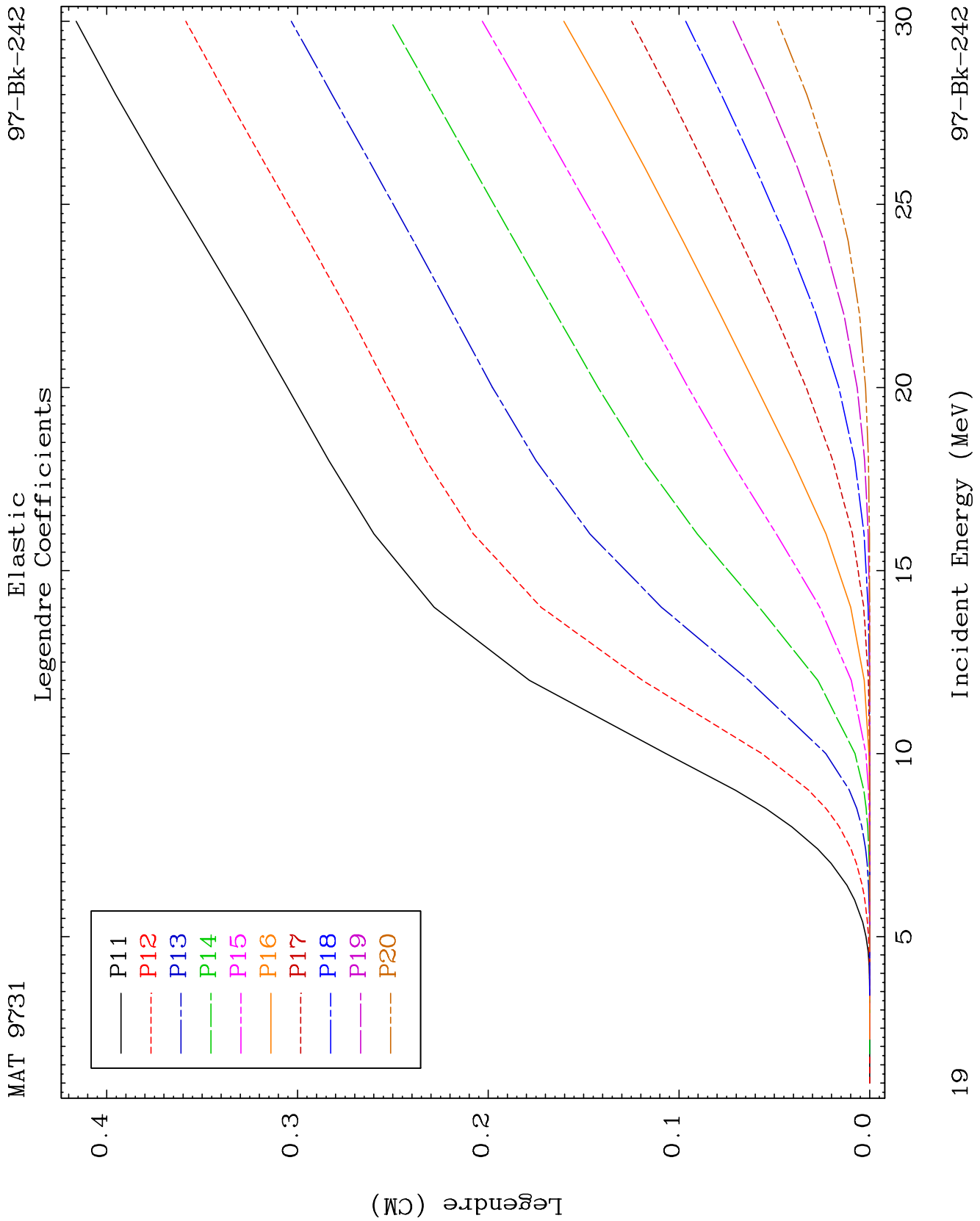
Energy Release
Parameters

97-Bk-242





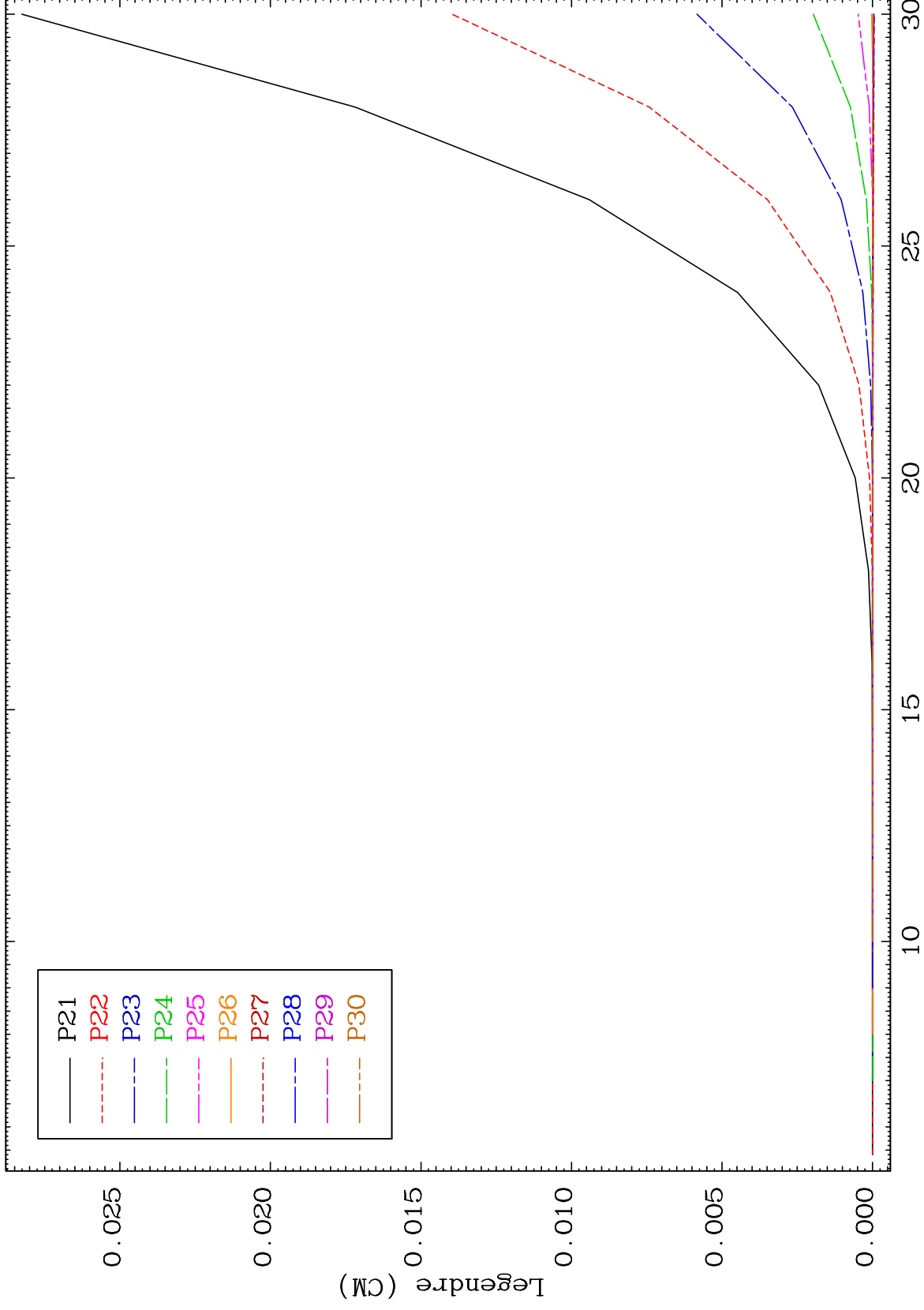




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Elastic Legendre Coefficients

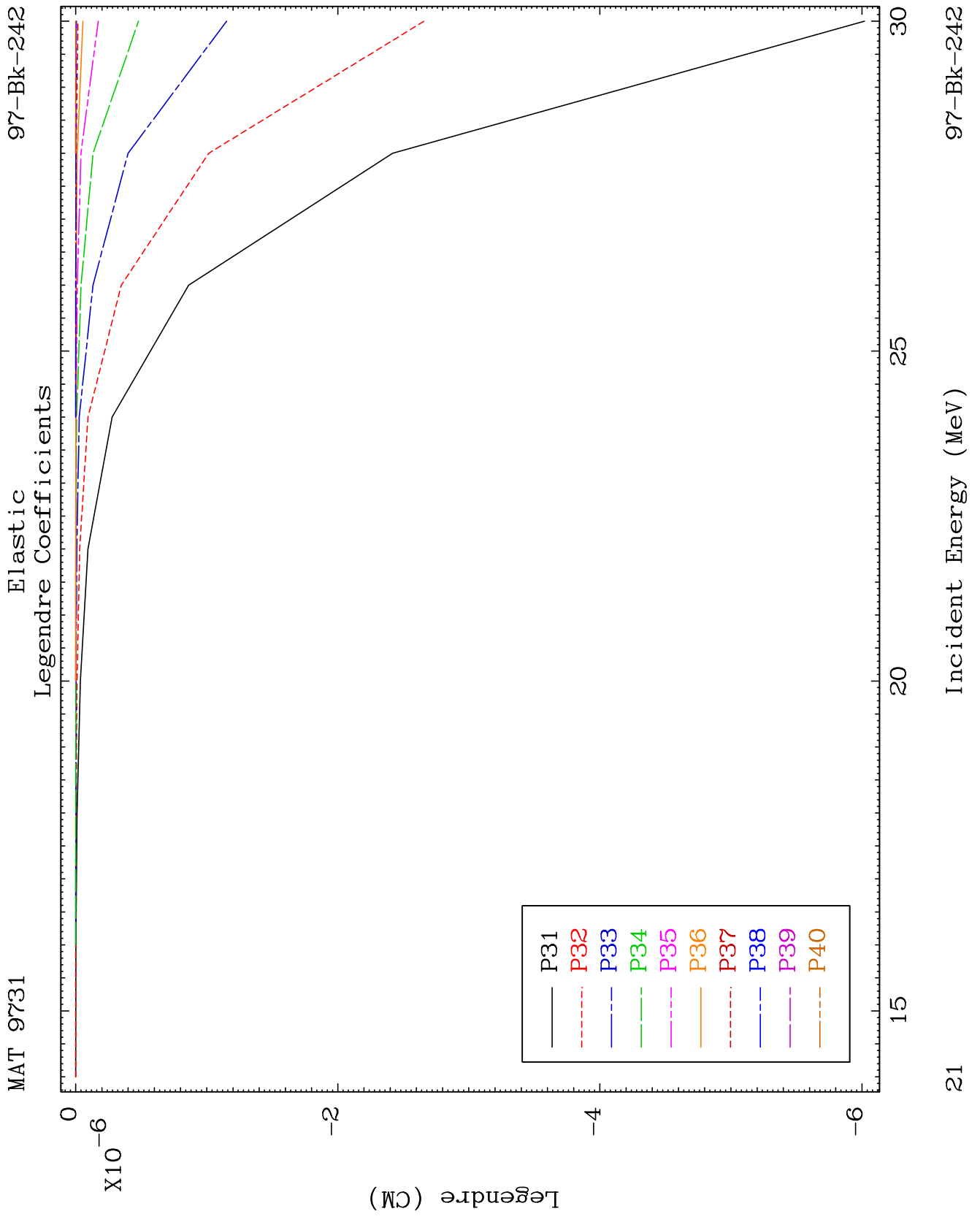
97-Bk-242



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Incident Energy (MeV)

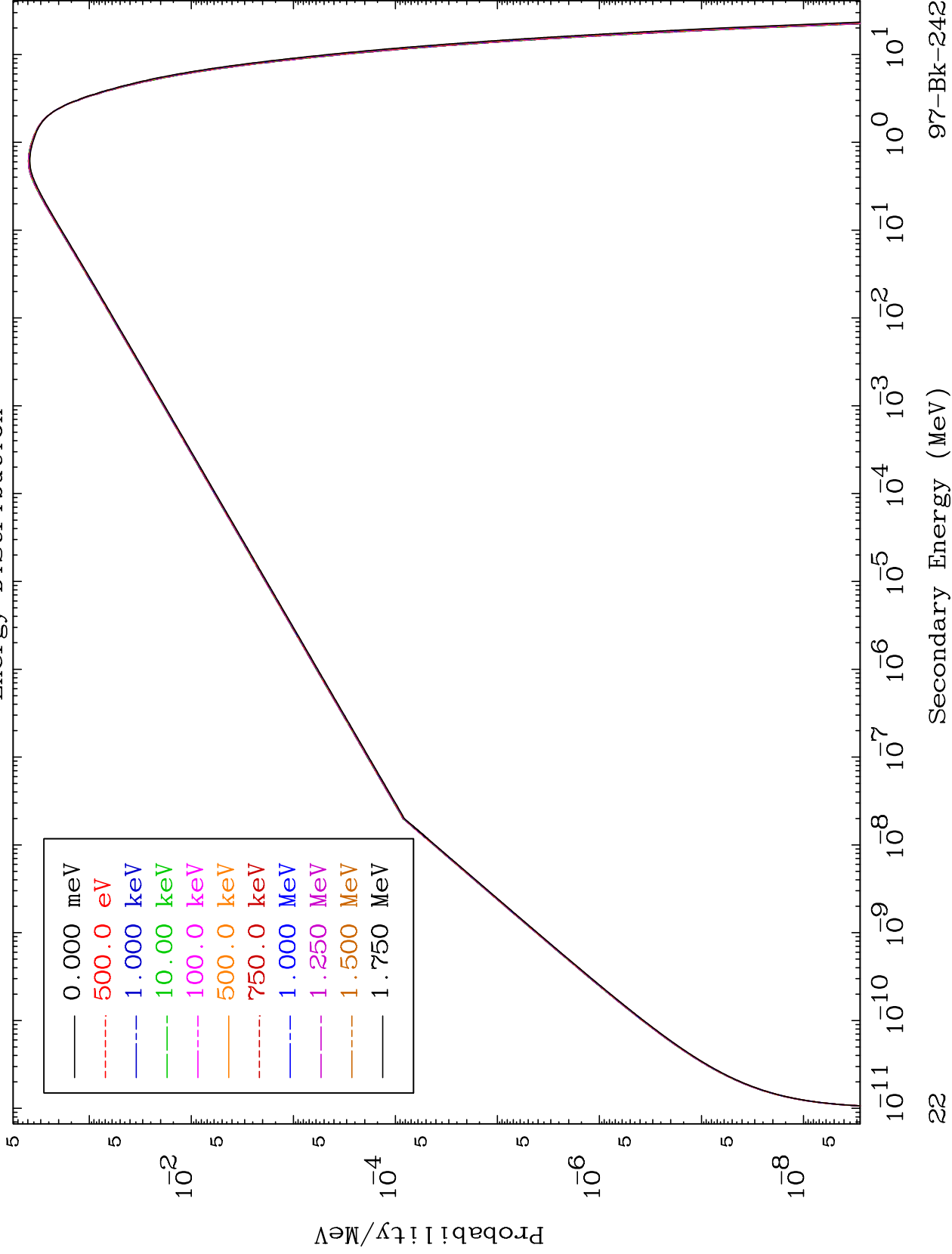
97-Bk-242



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Fission Energy Distribution

97-Bk-242



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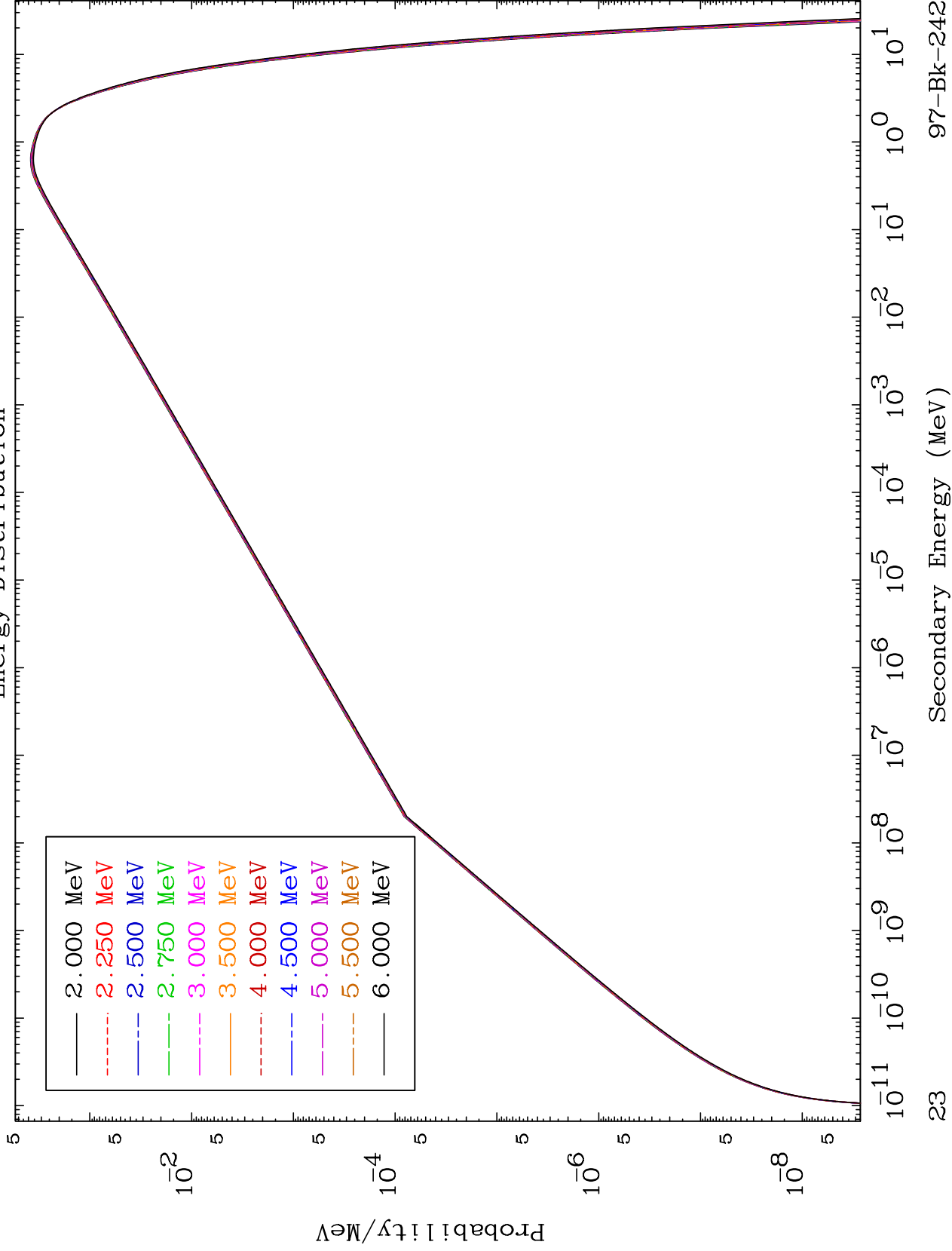
Secondary Energy (MeV)

97-Bk-242

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Fission Energy Distribution

97-Bk-242



97-Bk-242

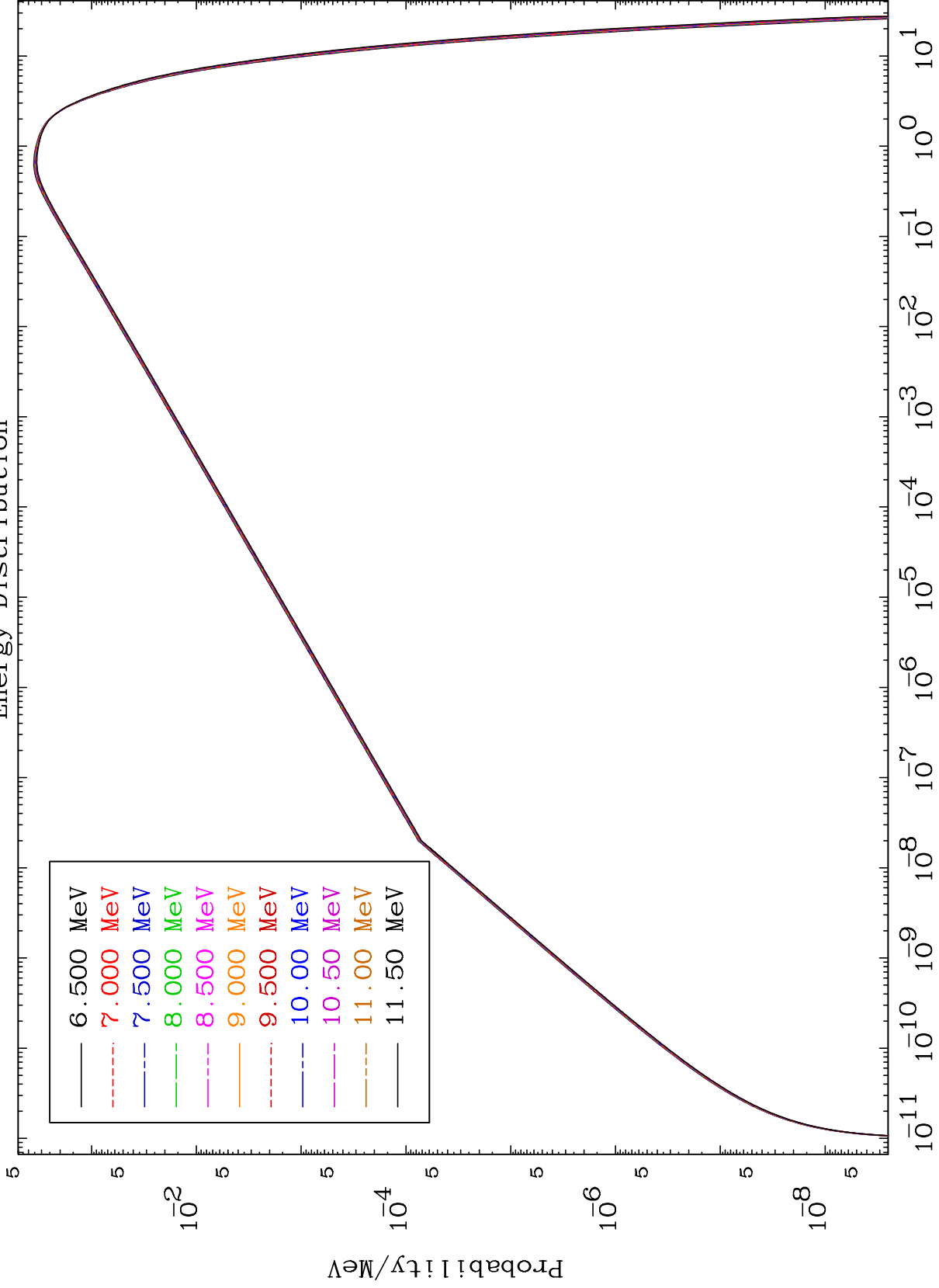
Secondary Energy (MeV)

23

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Fission Energy Distribution

97-Bk-242



24

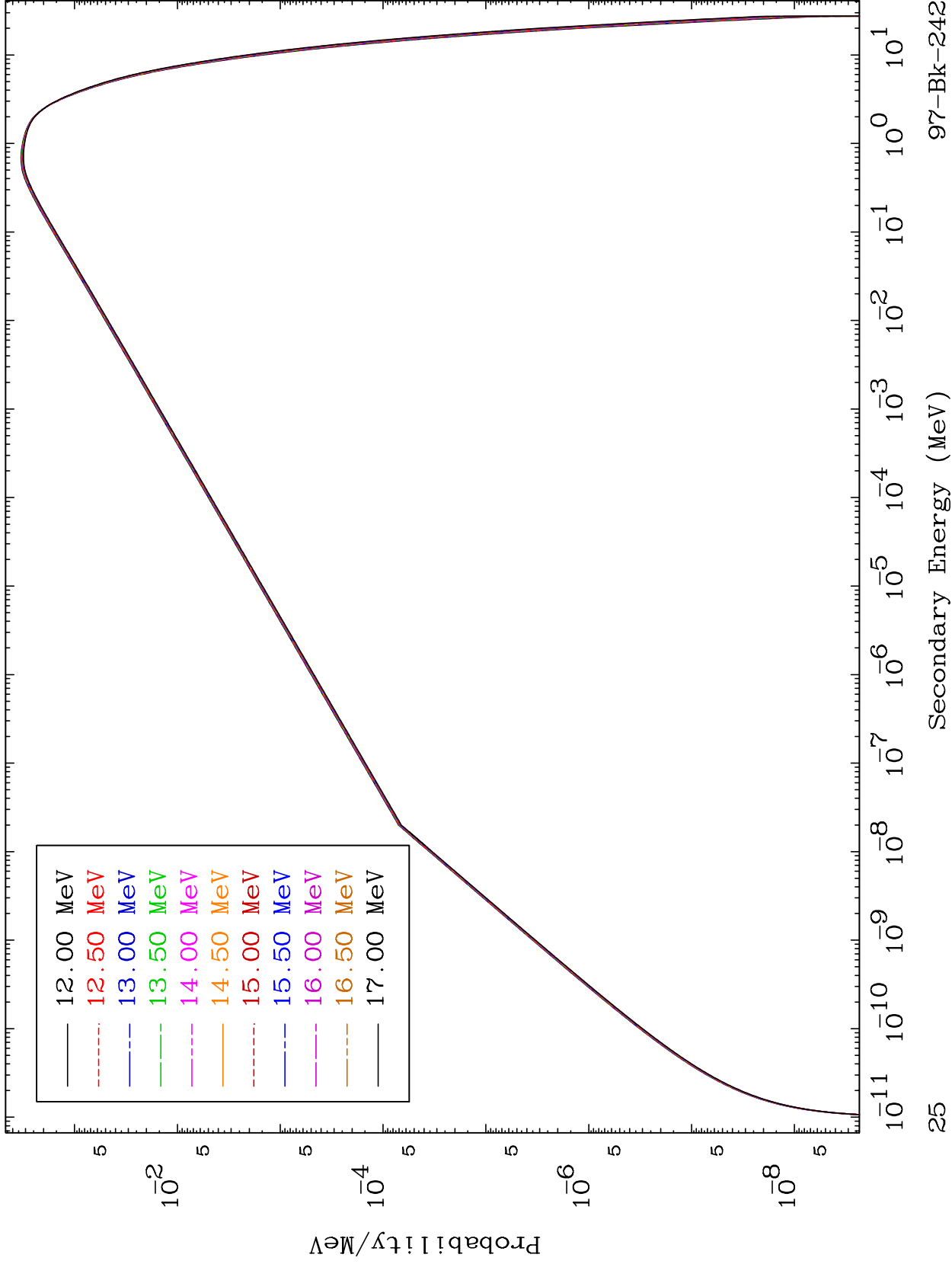
Secondary Energy (MeV)

97-Bk-242

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Fission Energy Distribution

97-Bk-242



25

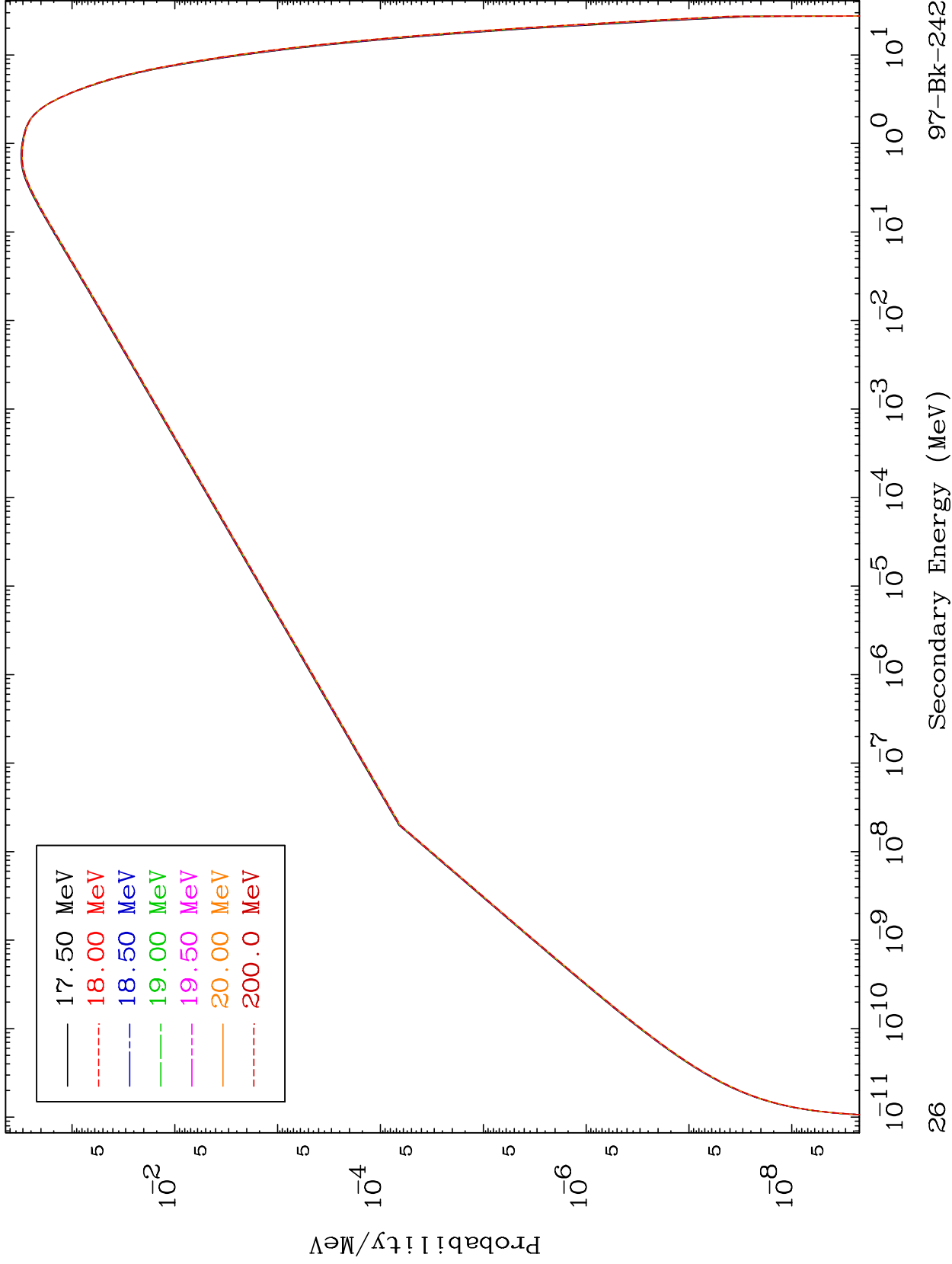
Secondary Energy (MeV)

97-Bk-242

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Fission
Energy Distribution

97-Bk-242

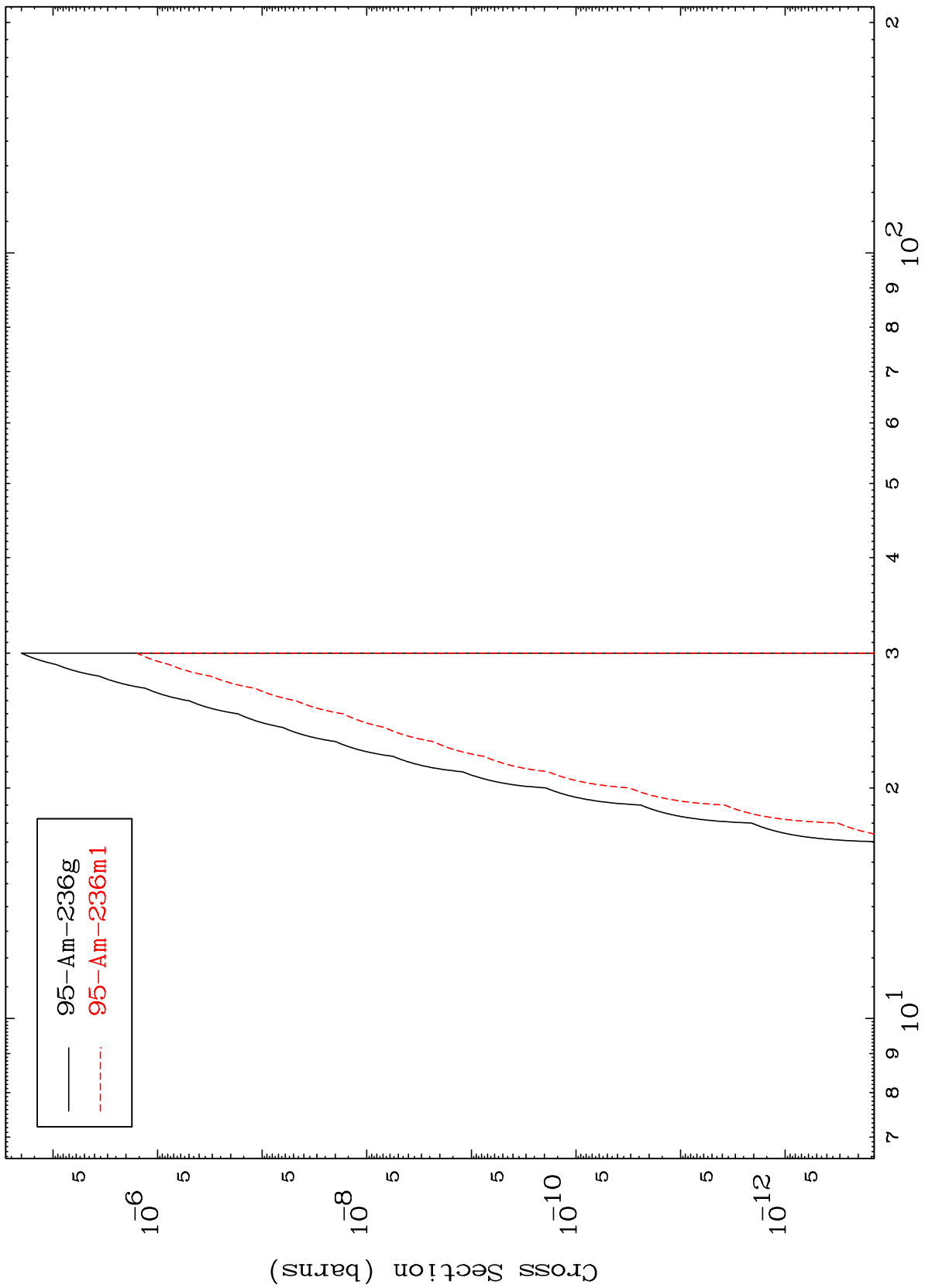


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(n,3n) α

97-Bk-242

Radionuclide Production Cross Section



27

Incident Energy (MeV)

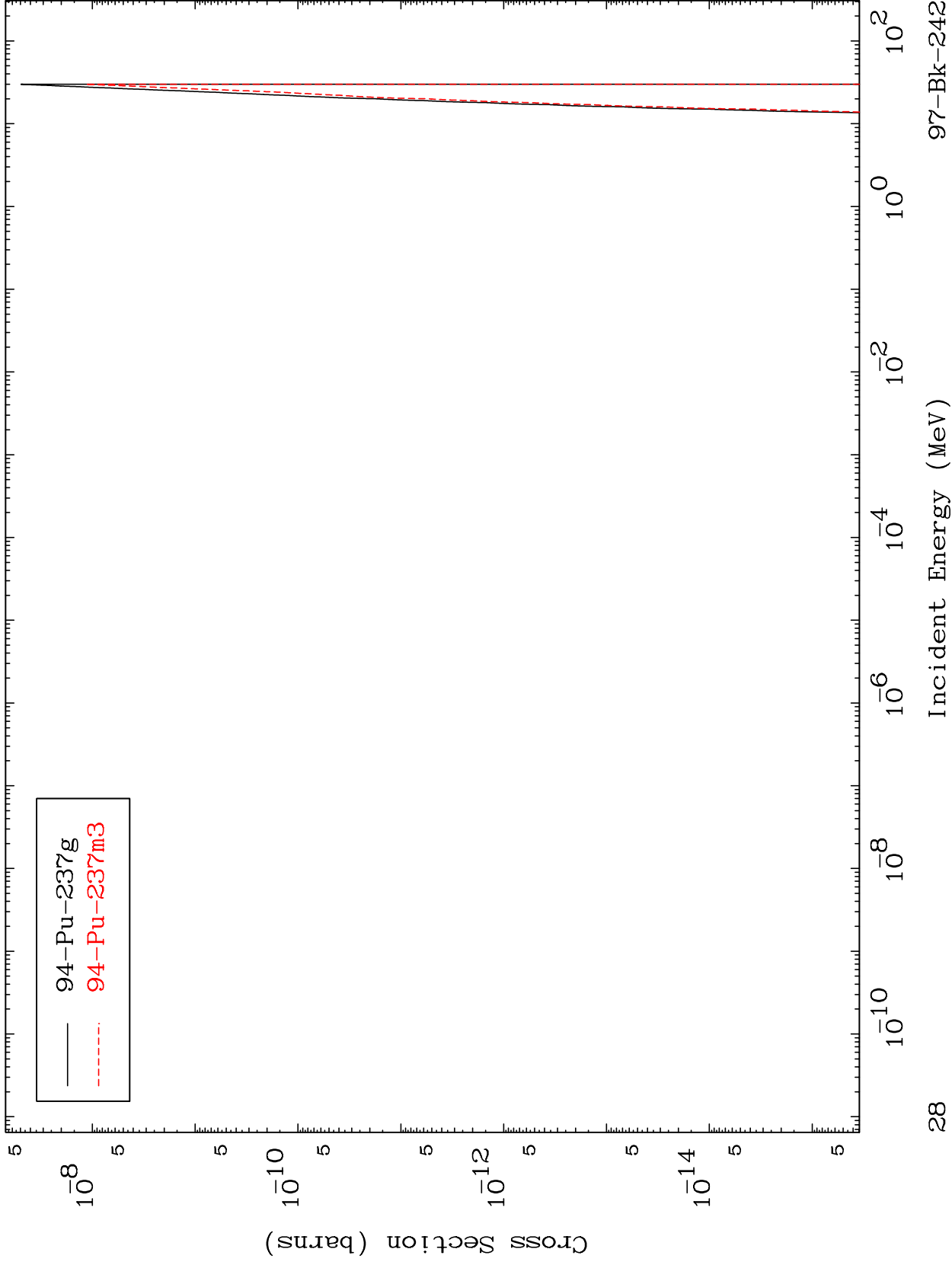
97-Bk-242

MAT 9731

(n,n') p α

97-Bk-242

Radionuclide Production Cross Section

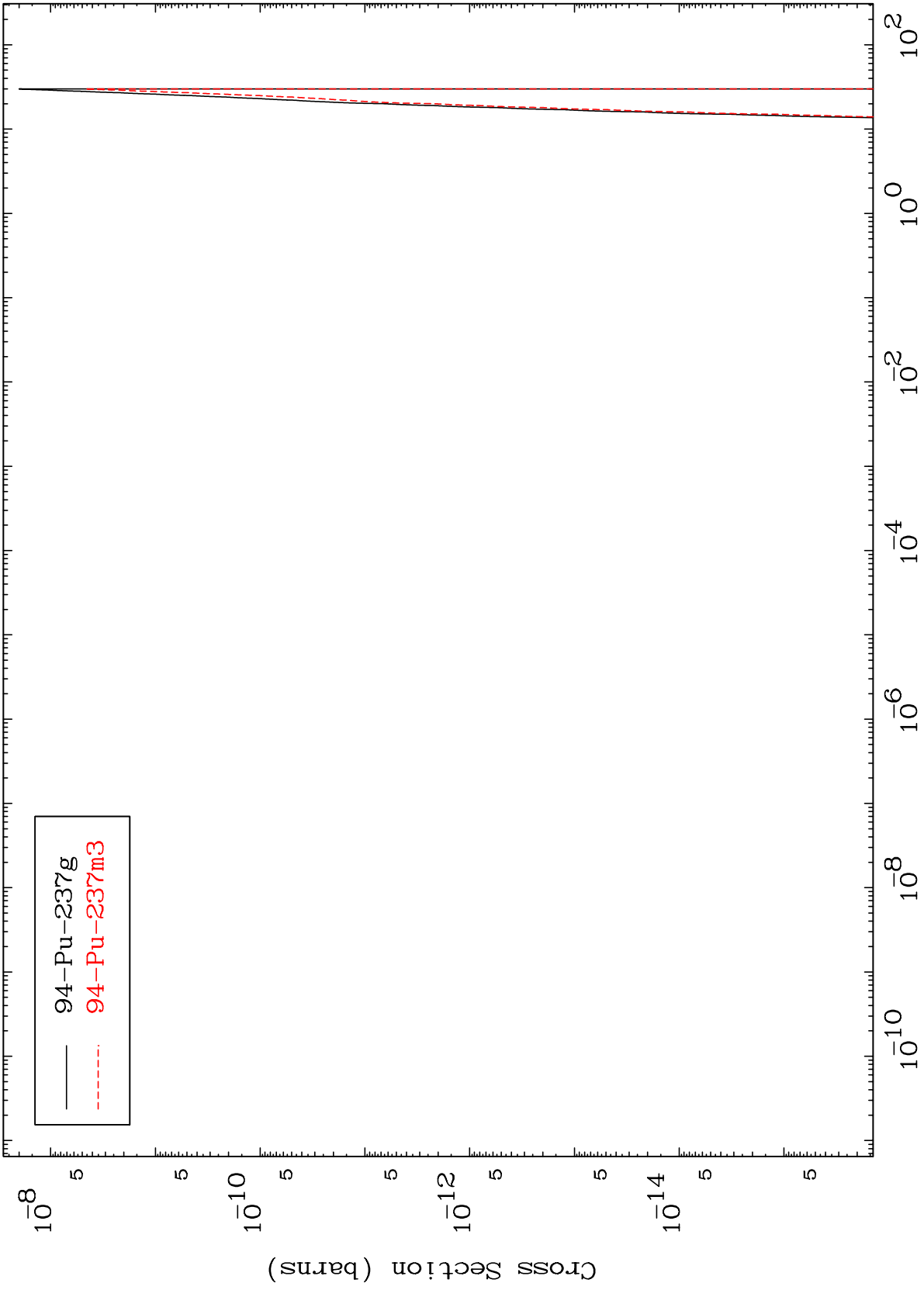


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(n,d) α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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