

Program EVALPLOT
(Version 2018-1)

by

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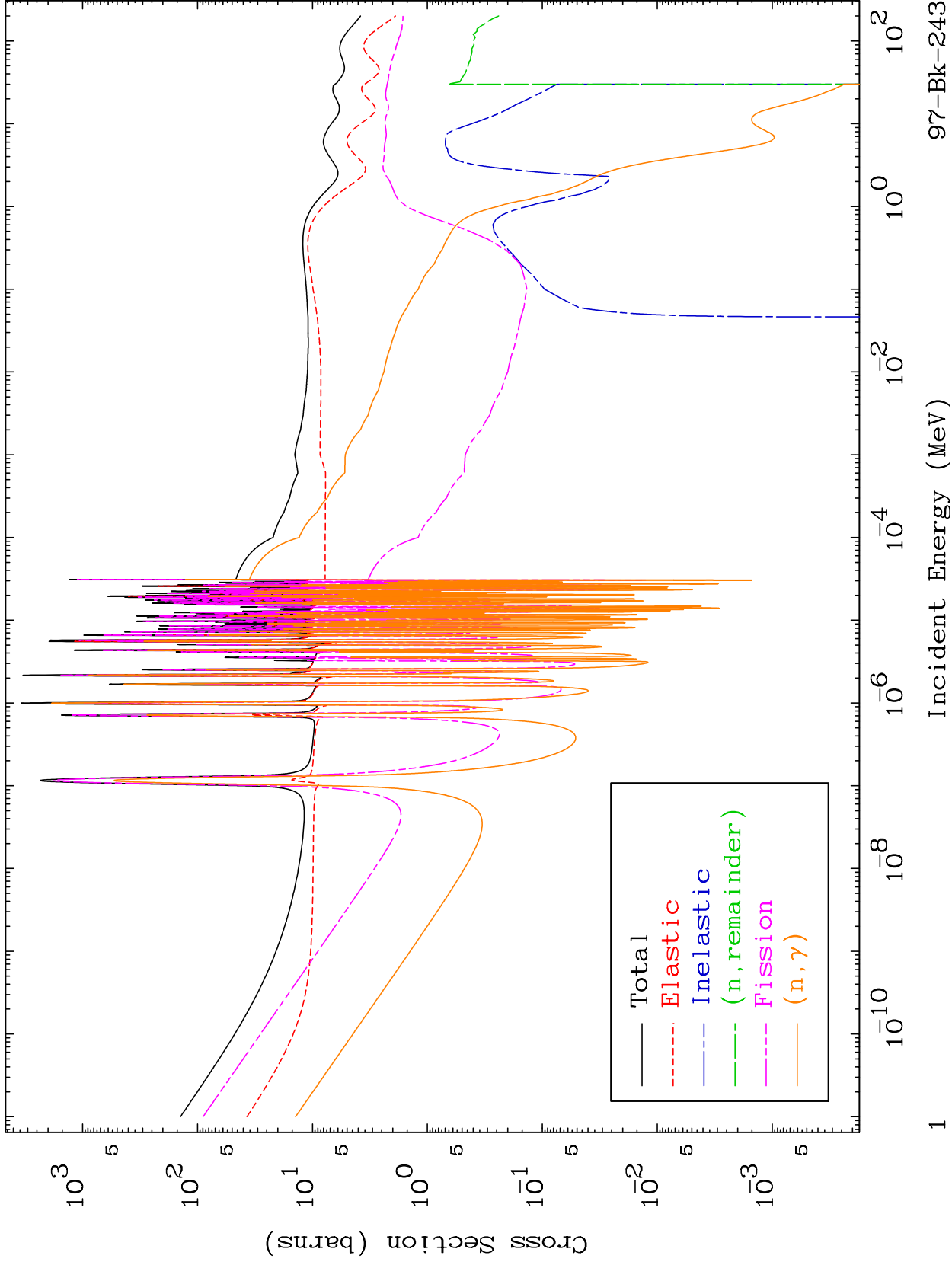
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Press Mouse Button to Start

MAT 9734

Major
293 Kelvin Cross Sections

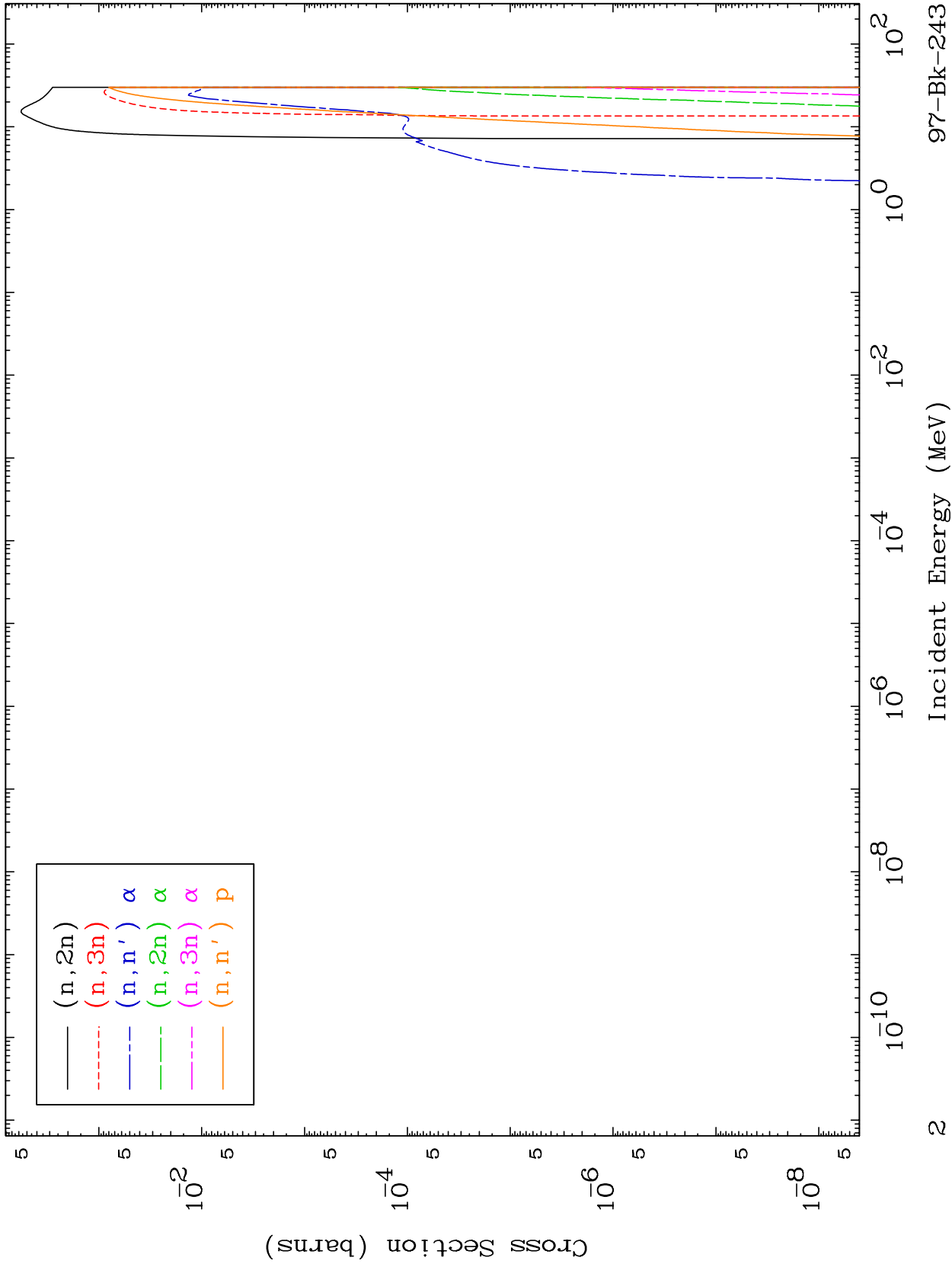
97-Bk-243



MAT 9734

Neutron Production
293 Kelvin Cross Sections

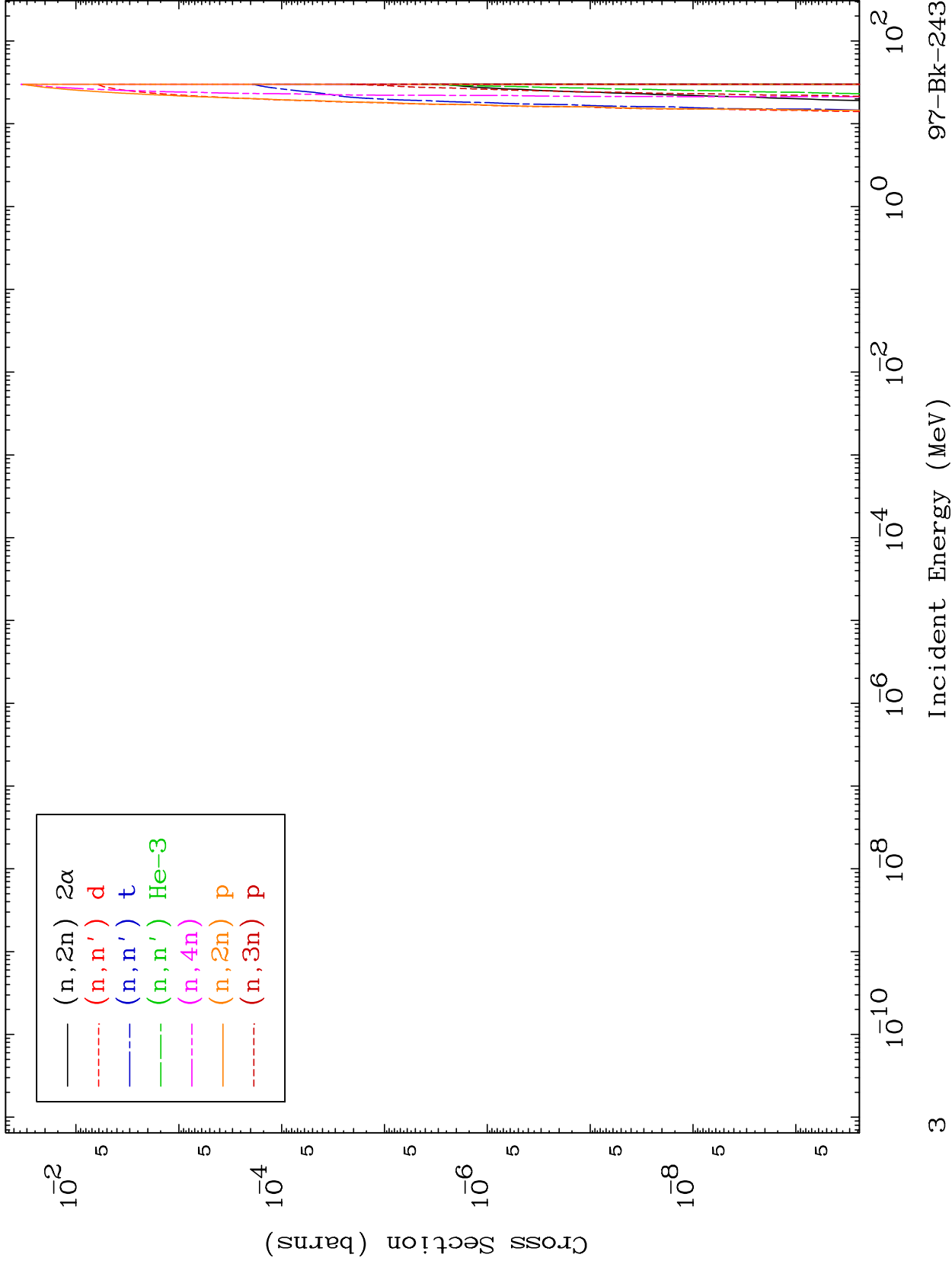
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MAT 9734

Neutron Production
293 Kelvin Cross Sections

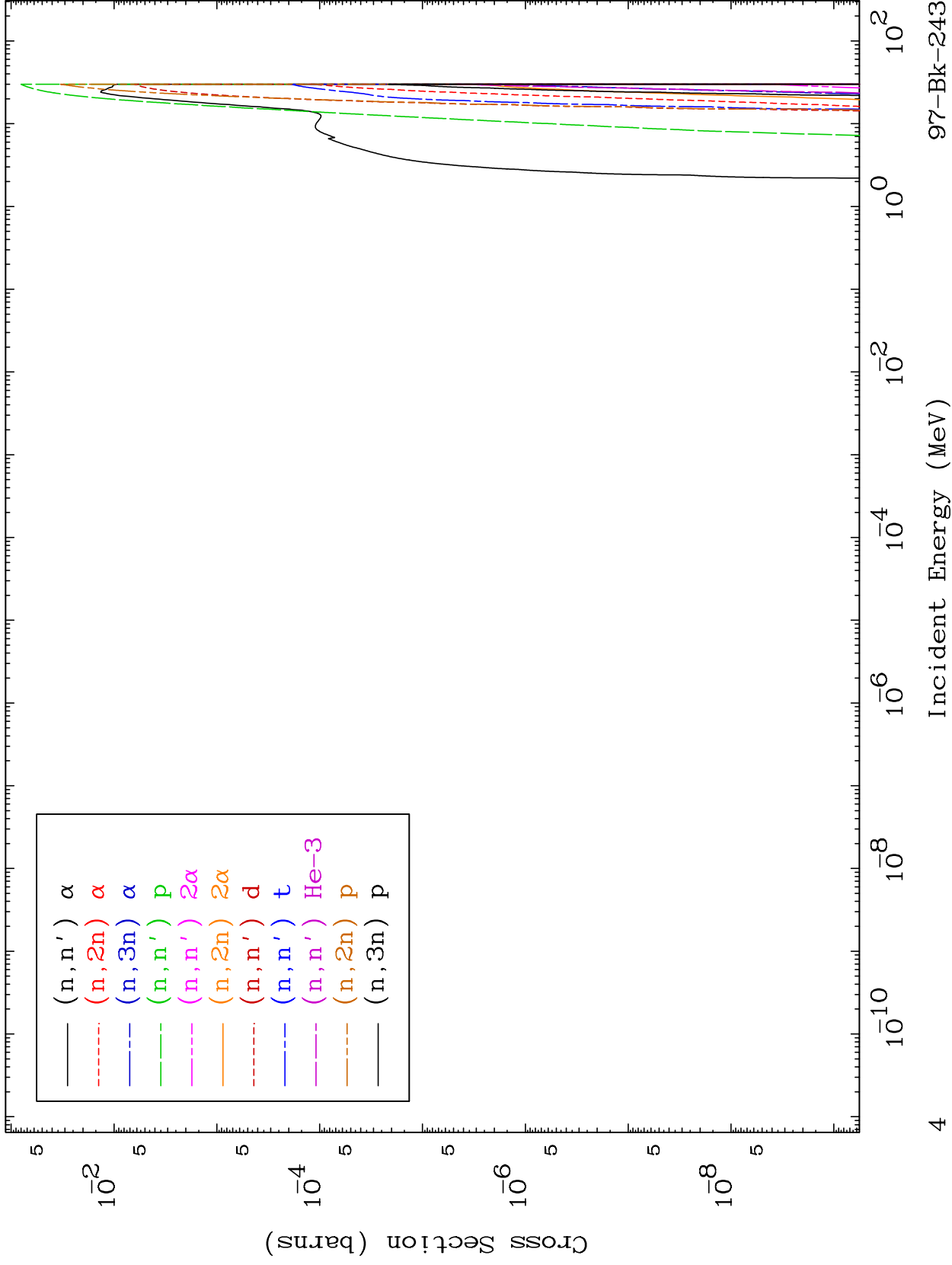
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MAT 9734

Charged Particle
293 Kelvin Cross Sections

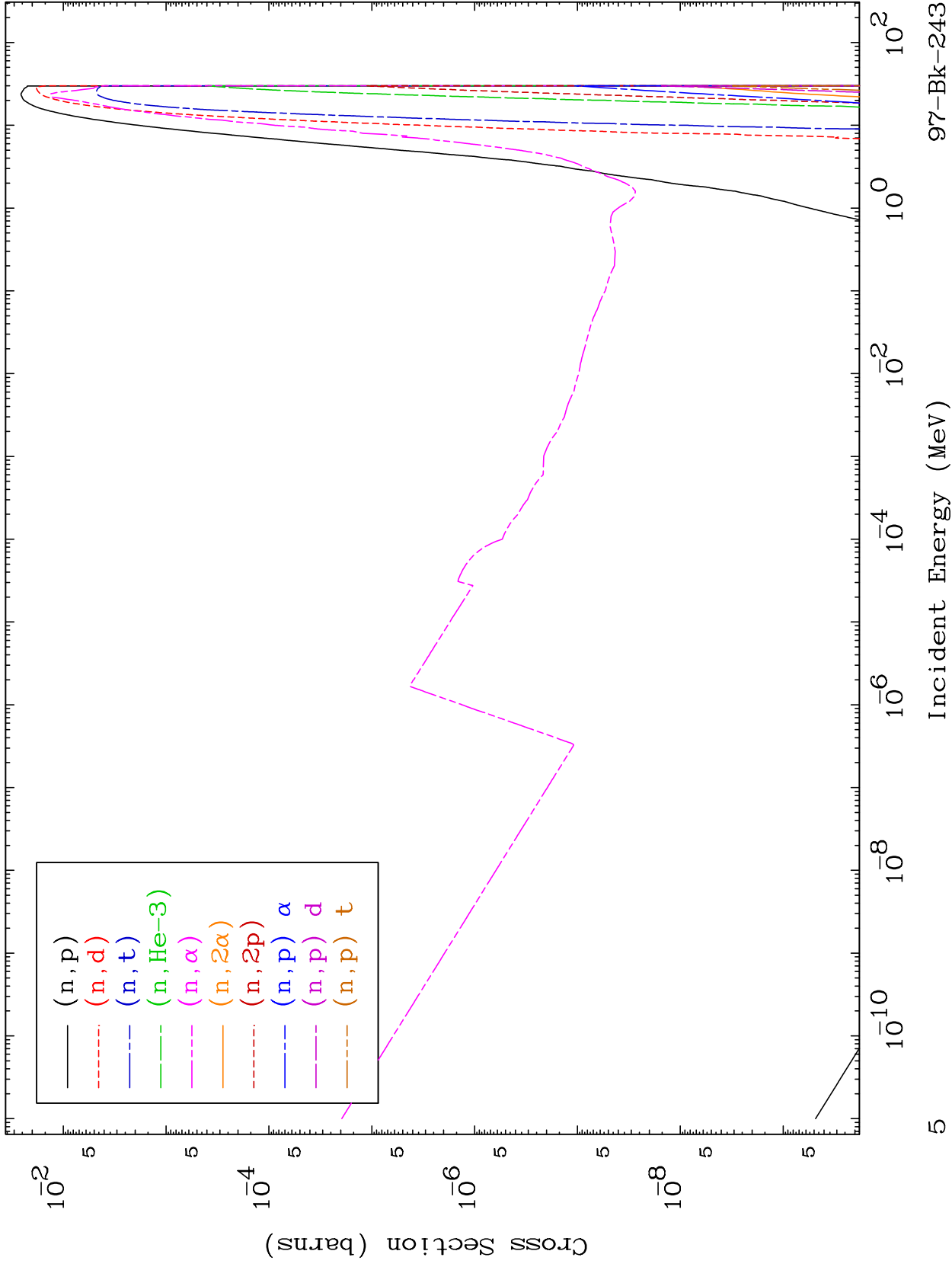
97-Bk-243



MAT 9734

Charged Particle
293 Kelvin Cross Sections

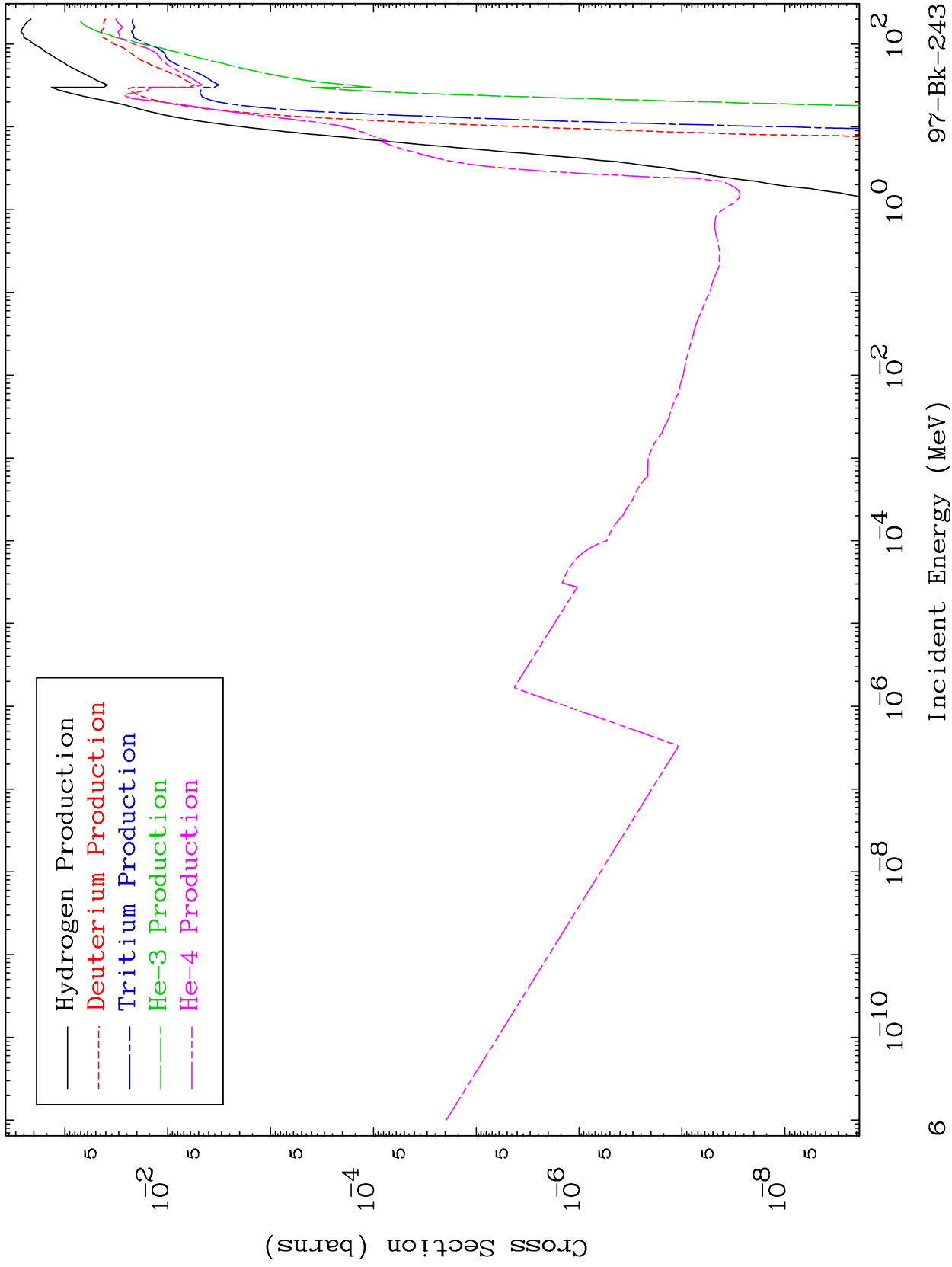
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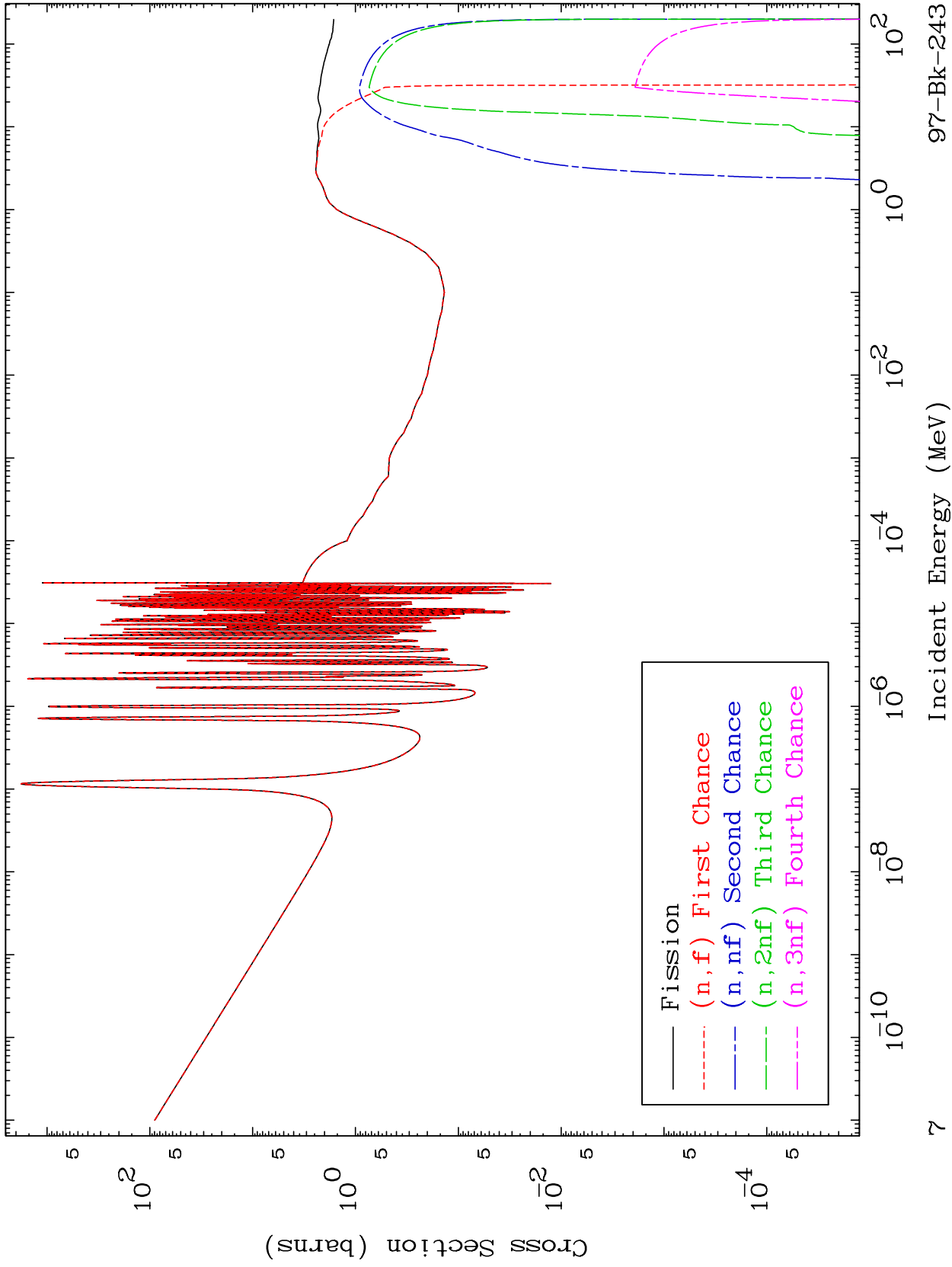


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Particle Production
293 Kelvin Cross Sections

97-Bk-243



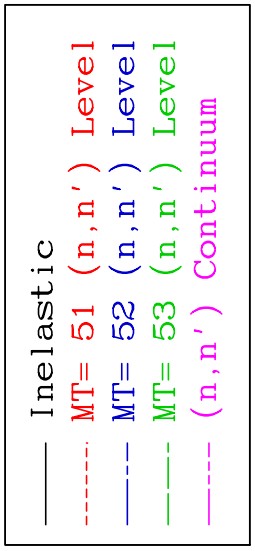
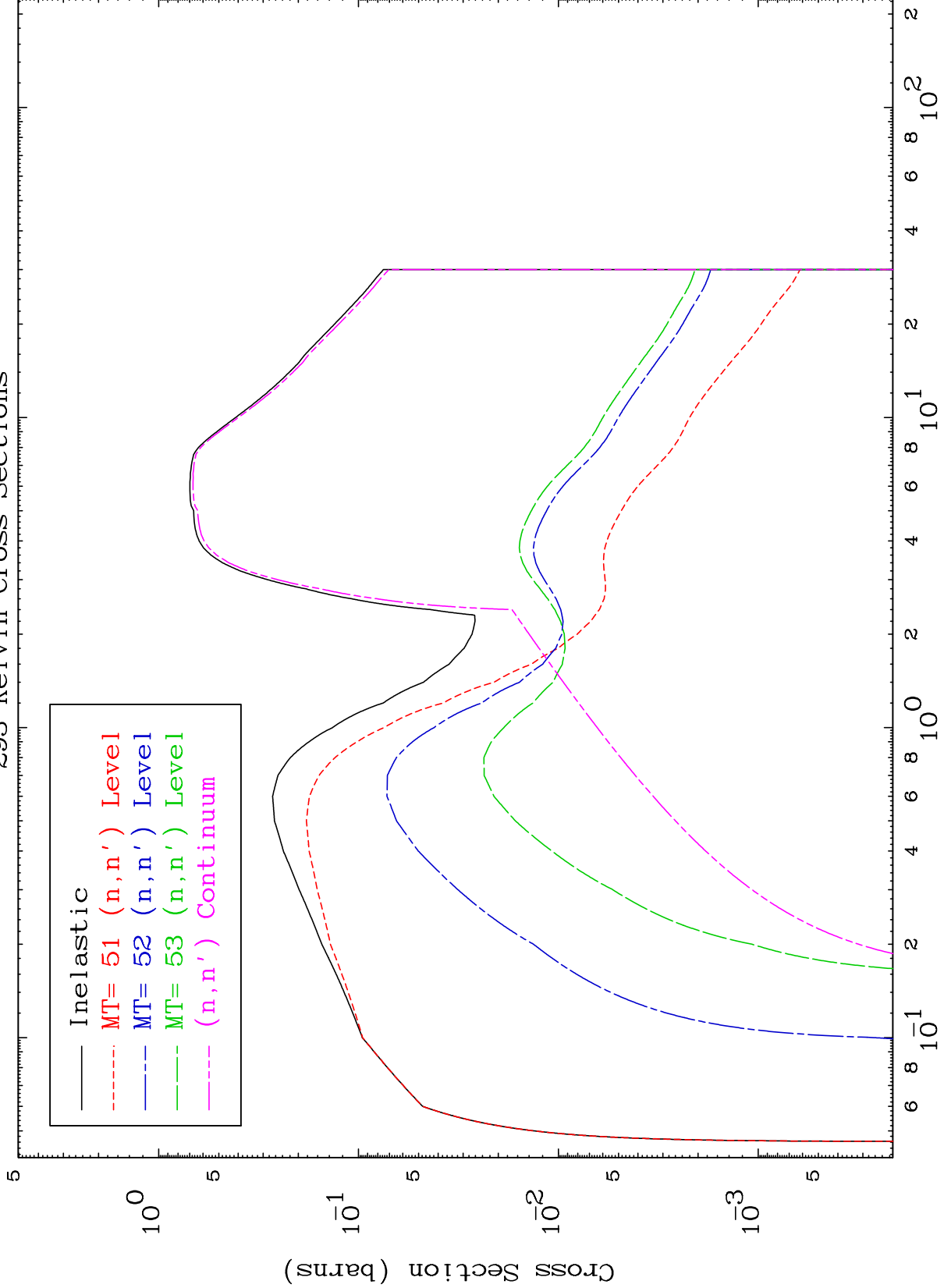


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(n,n') Level

97-Bk-243

293 Kelvin Cross Sections



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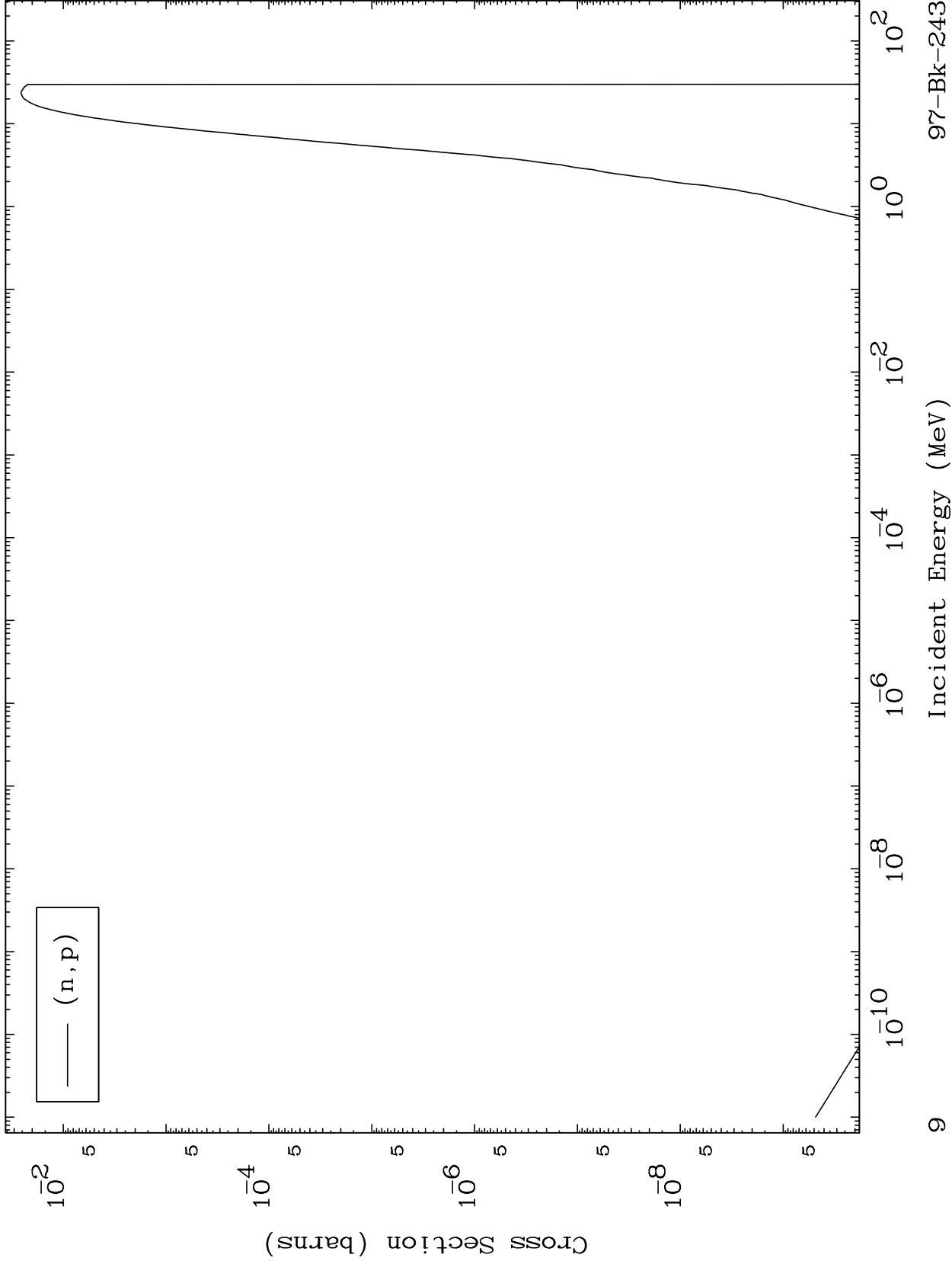
Incident Energy (MeV)

97-Bk-243

MAT 9734

(n,p) Levels
293 Kelvin Cross Sections

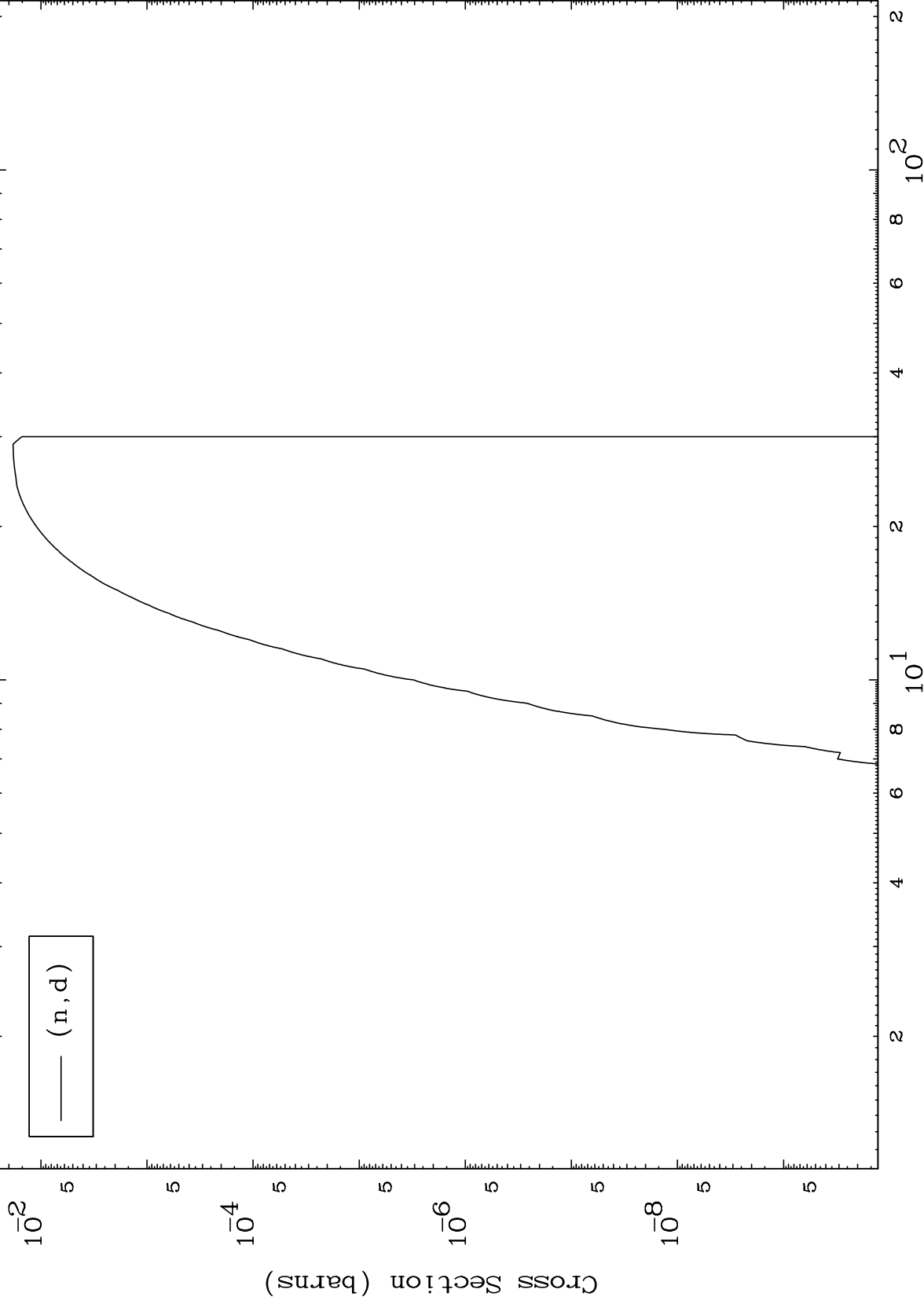
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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-243



10

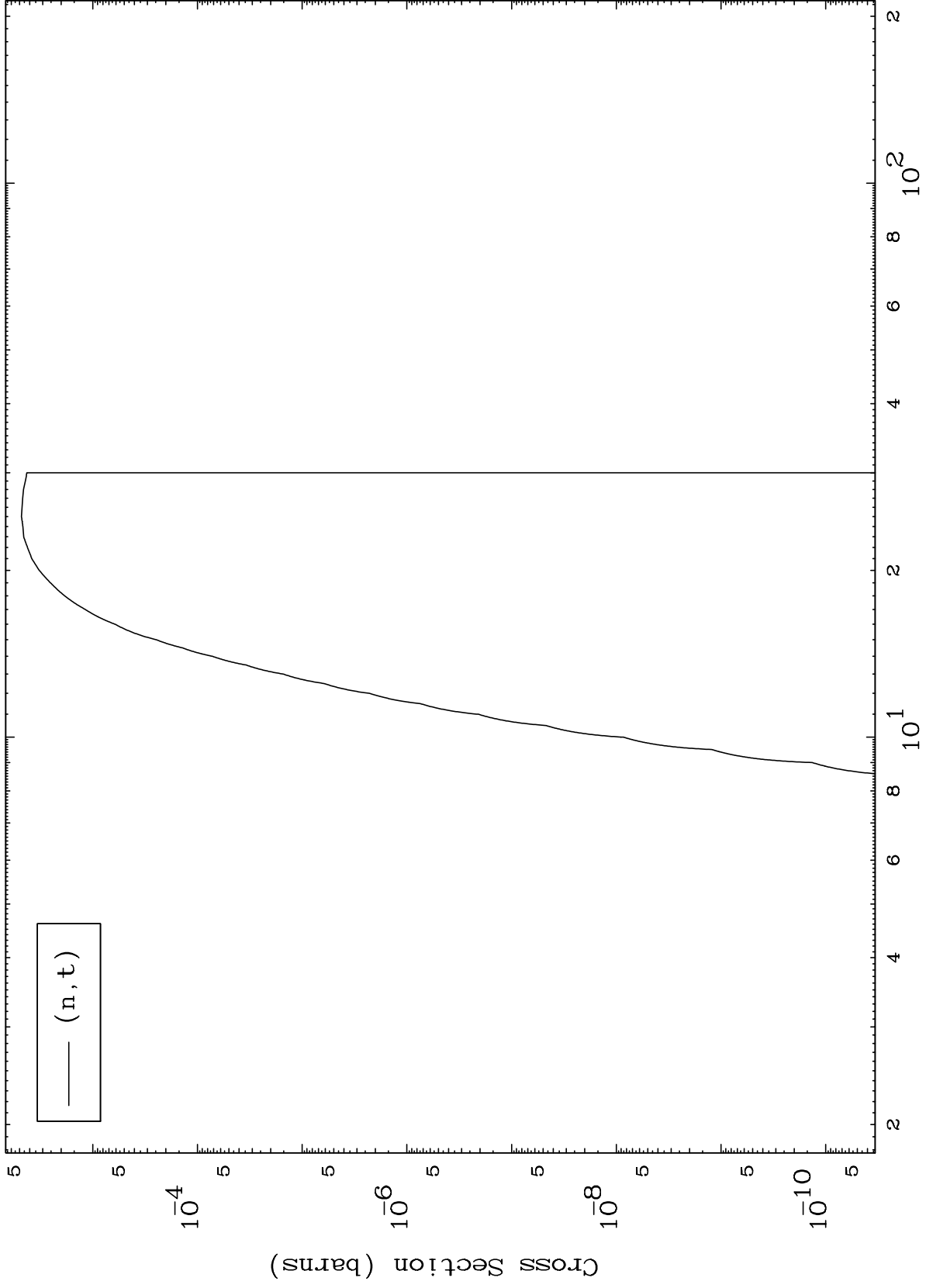
Incident Energy (MeV)

97-Bk-243

MAT 9734

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-243



11

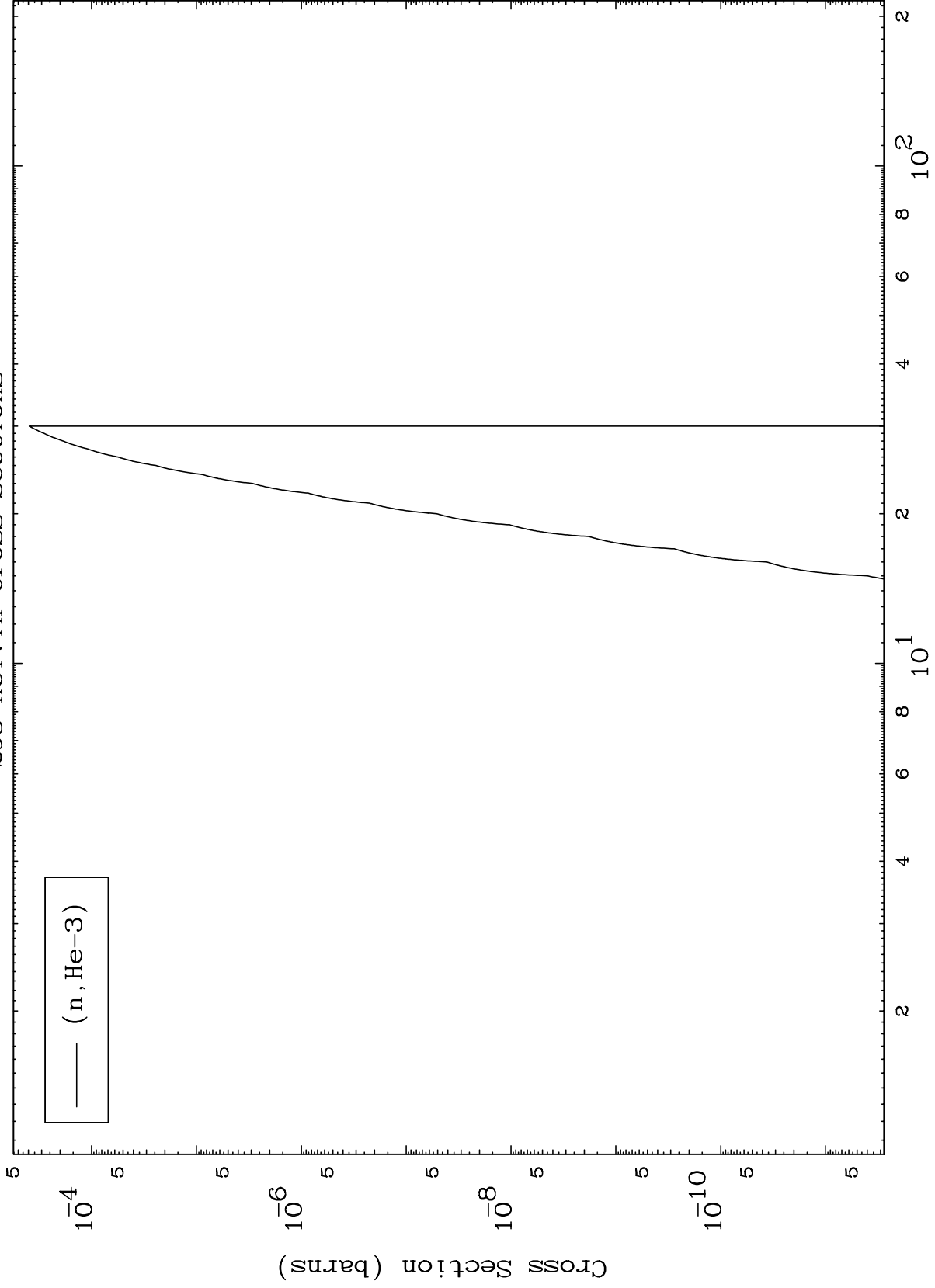
Incident Energy (MeV)

97-Bk-243

MAT 9734

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-243



12

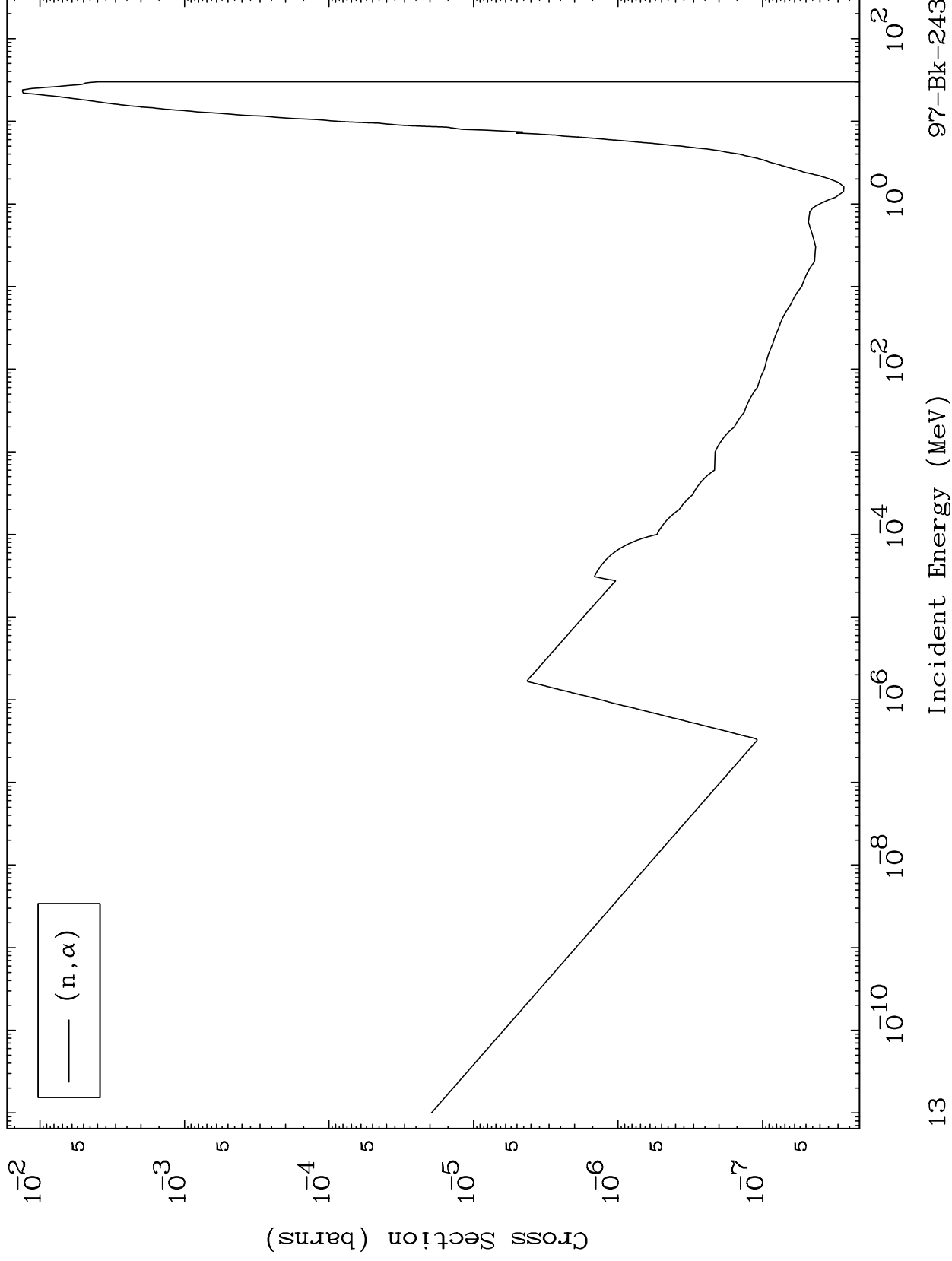
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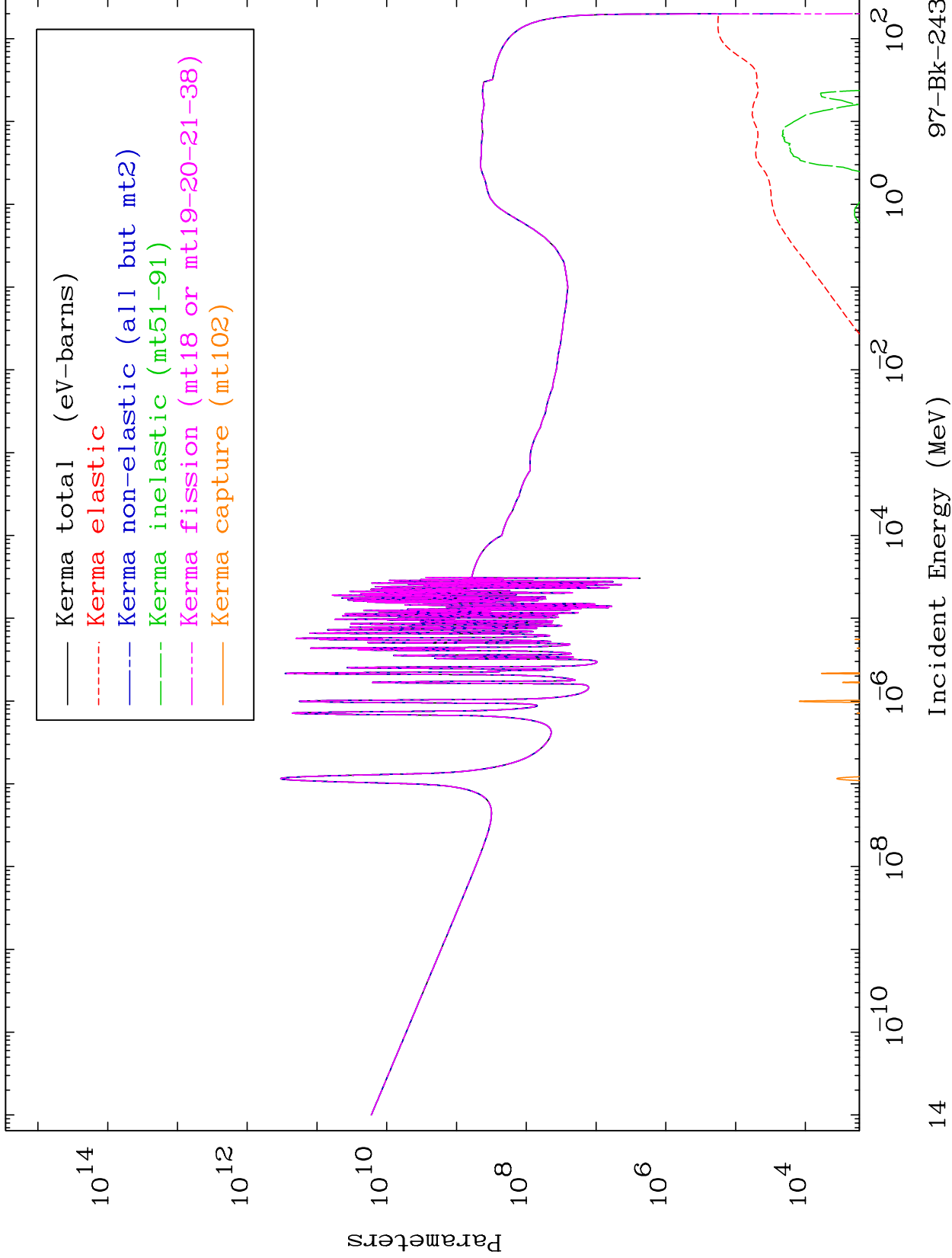
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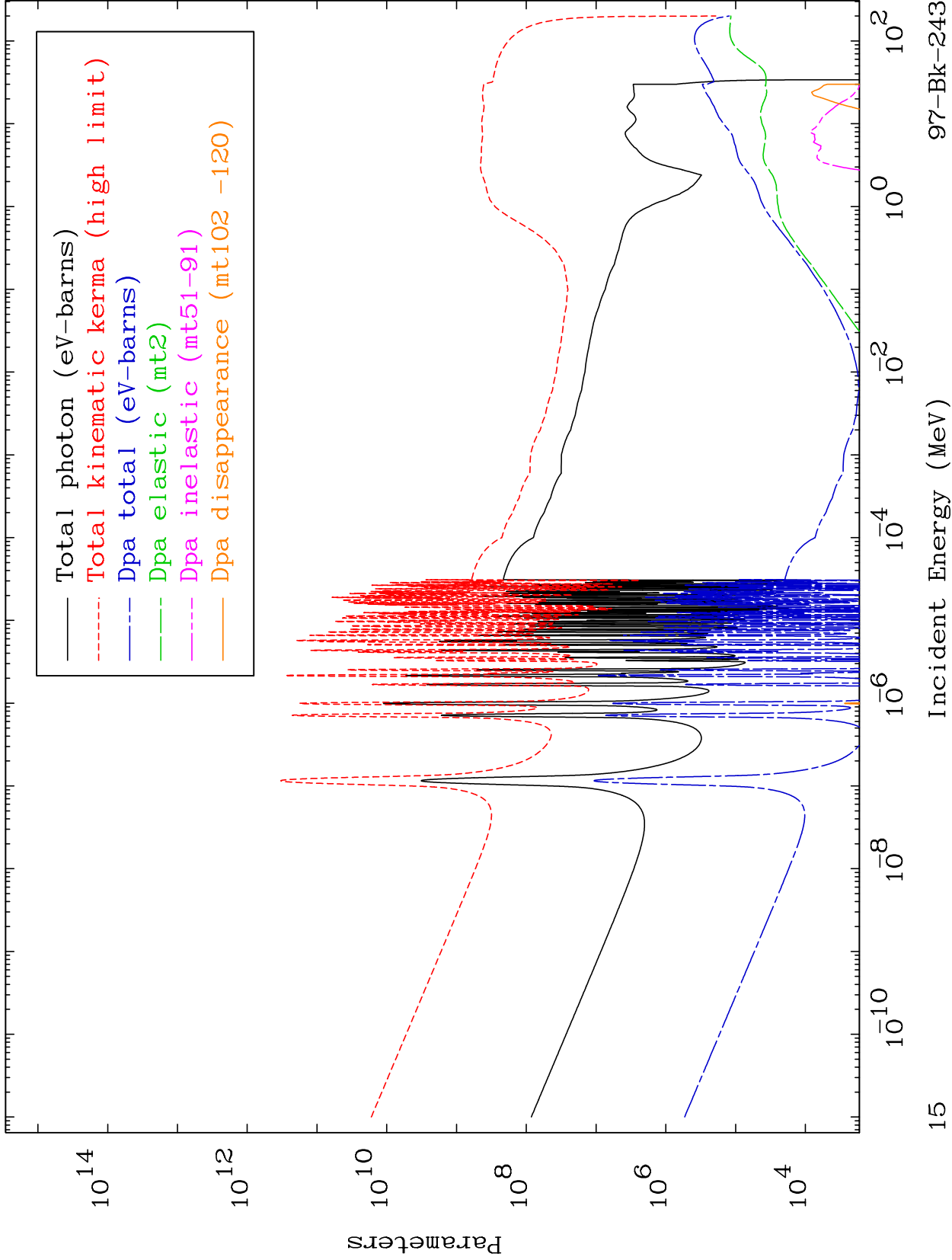
MAT 9734

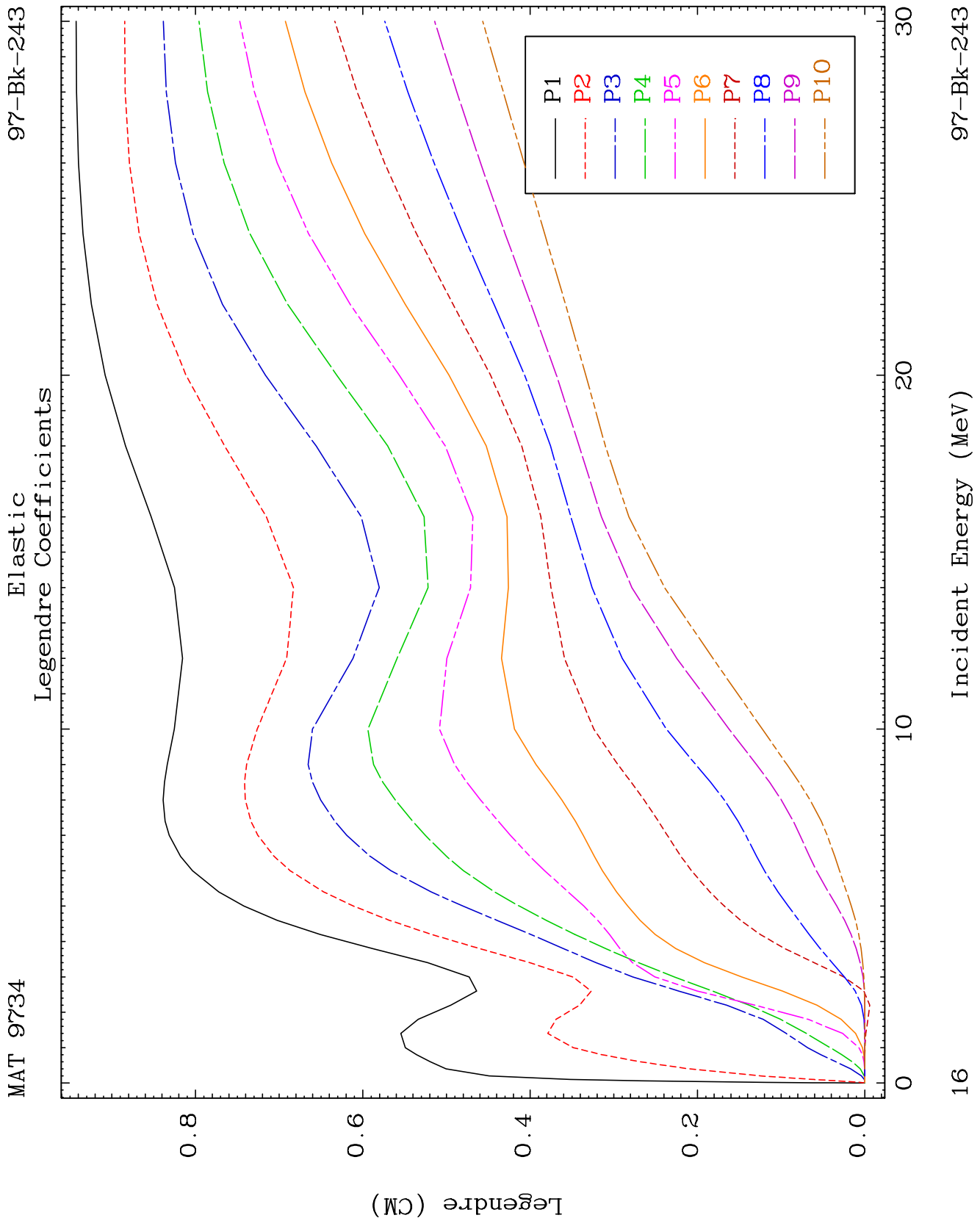
(n, α) Levels
293 Kelvin Cross Sections

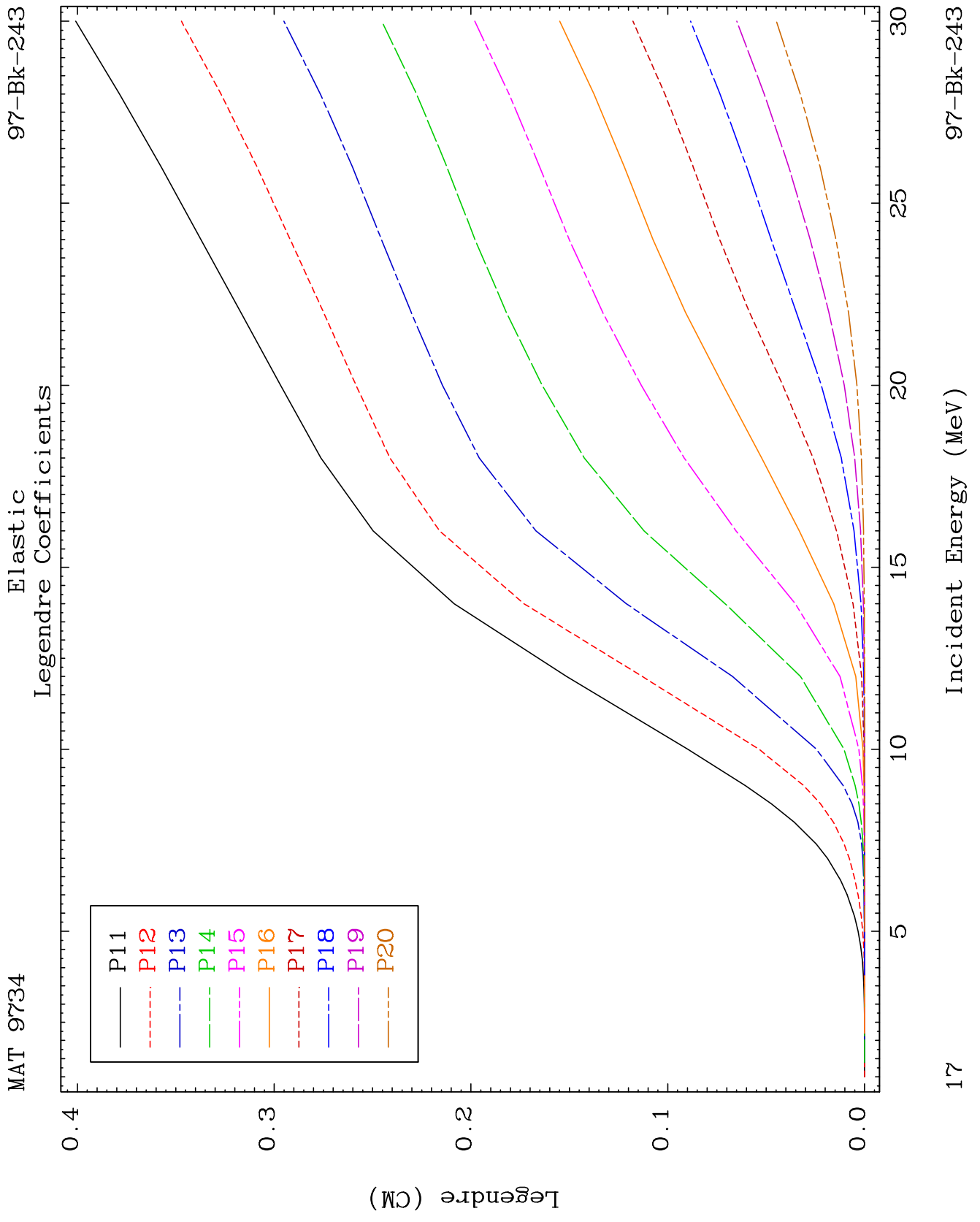
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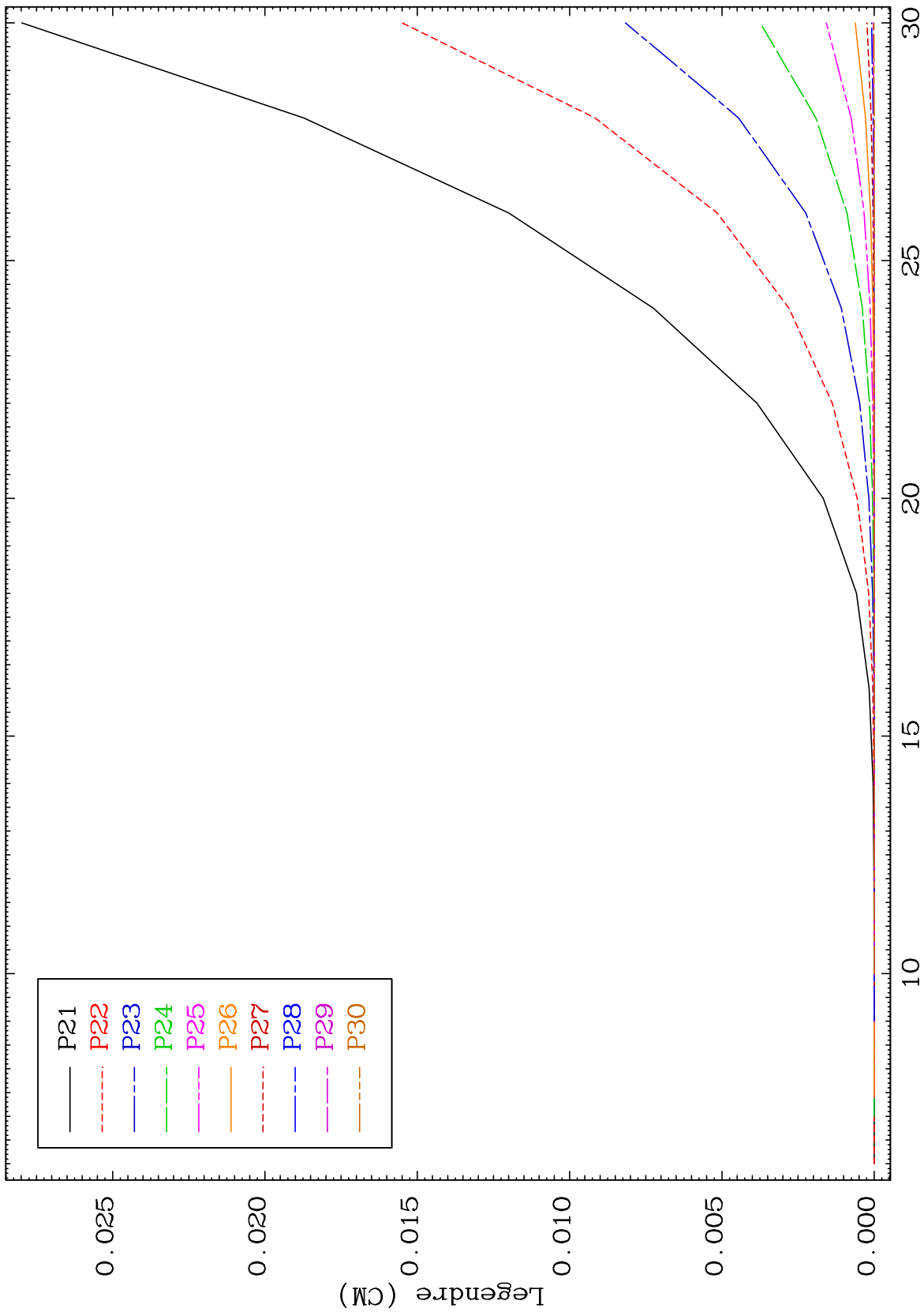




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Elastic Legendre Coefficients

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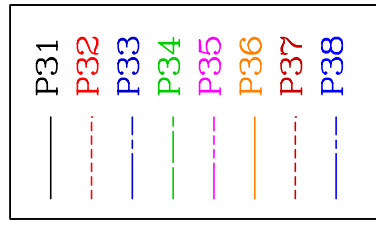
Incident Energy (MeV)

97-Bk-243

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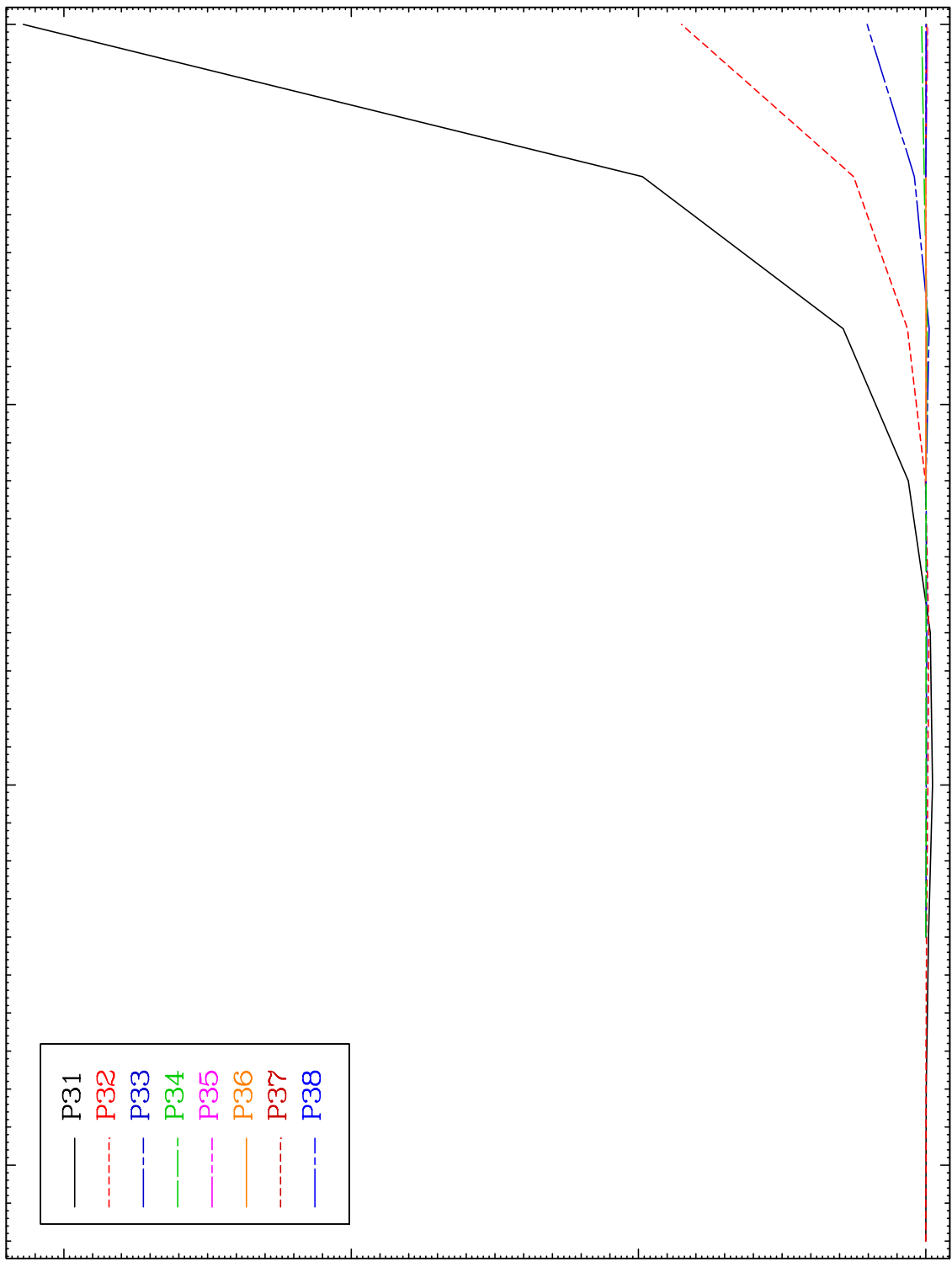
Elastic
Legendre Coefficients

97-Bk-243



$\times 10^{-63}$

Legendre (CM)

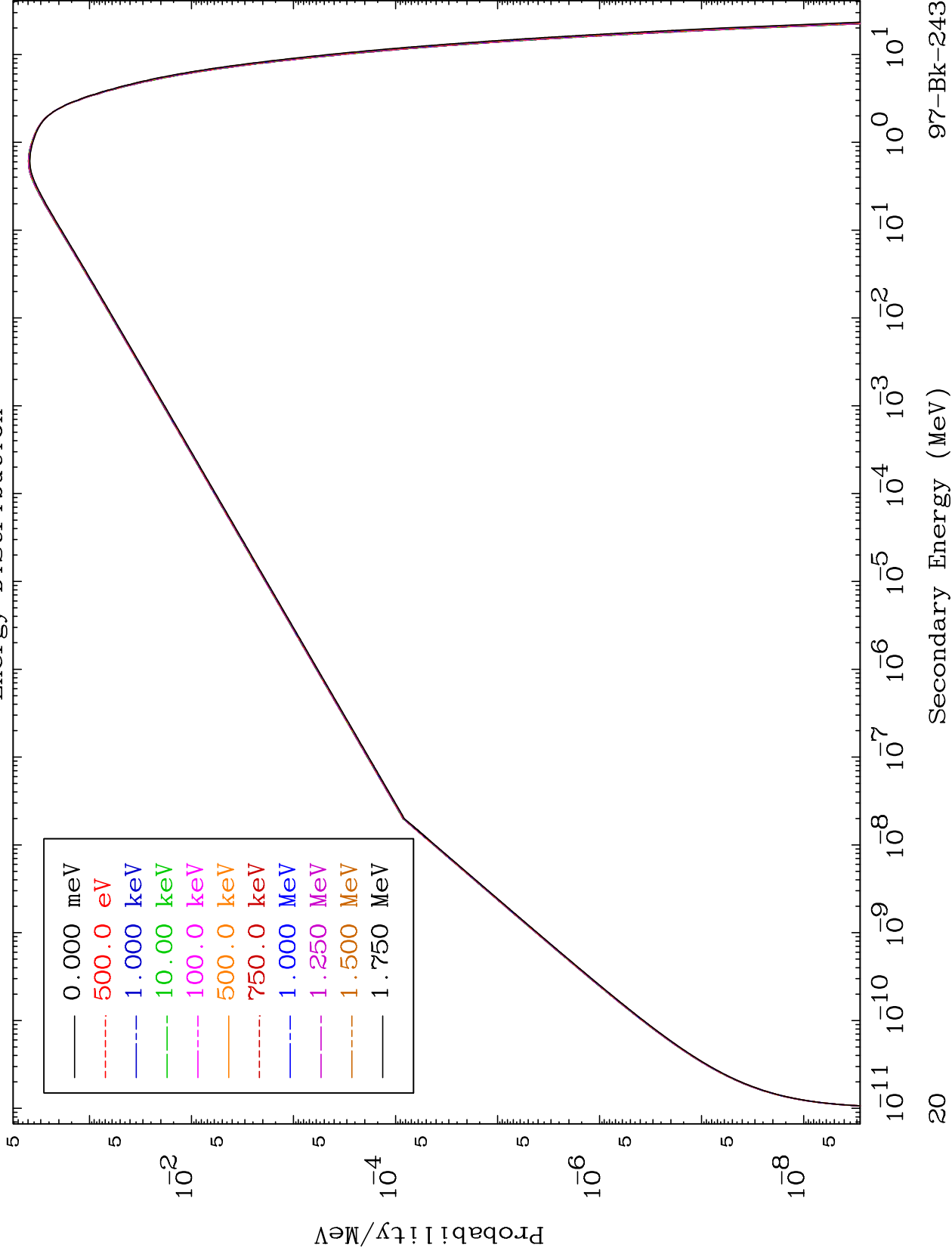


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Incident Energy (MeV)

97-Bk-243

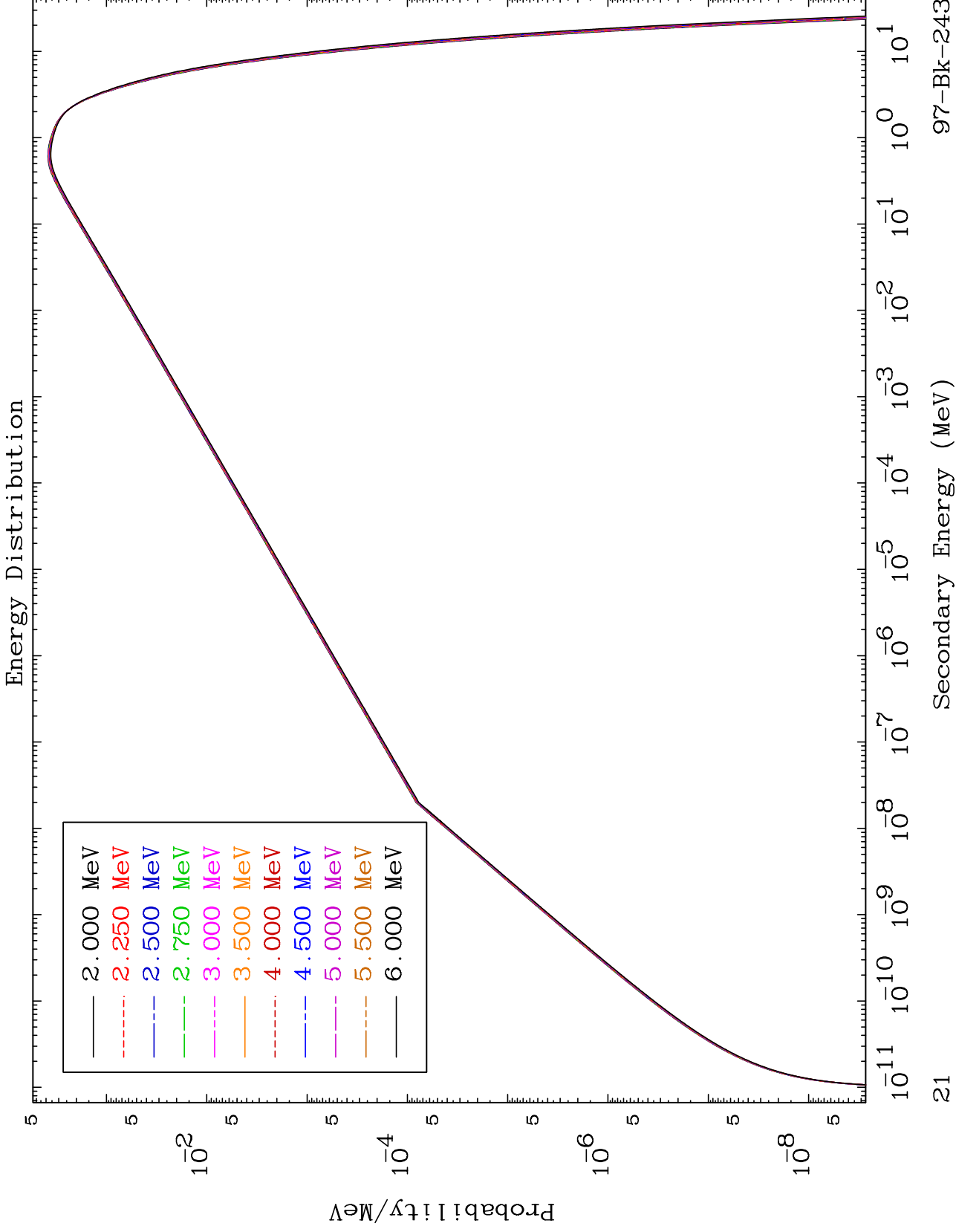
Fission Energy Distribution



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Fission

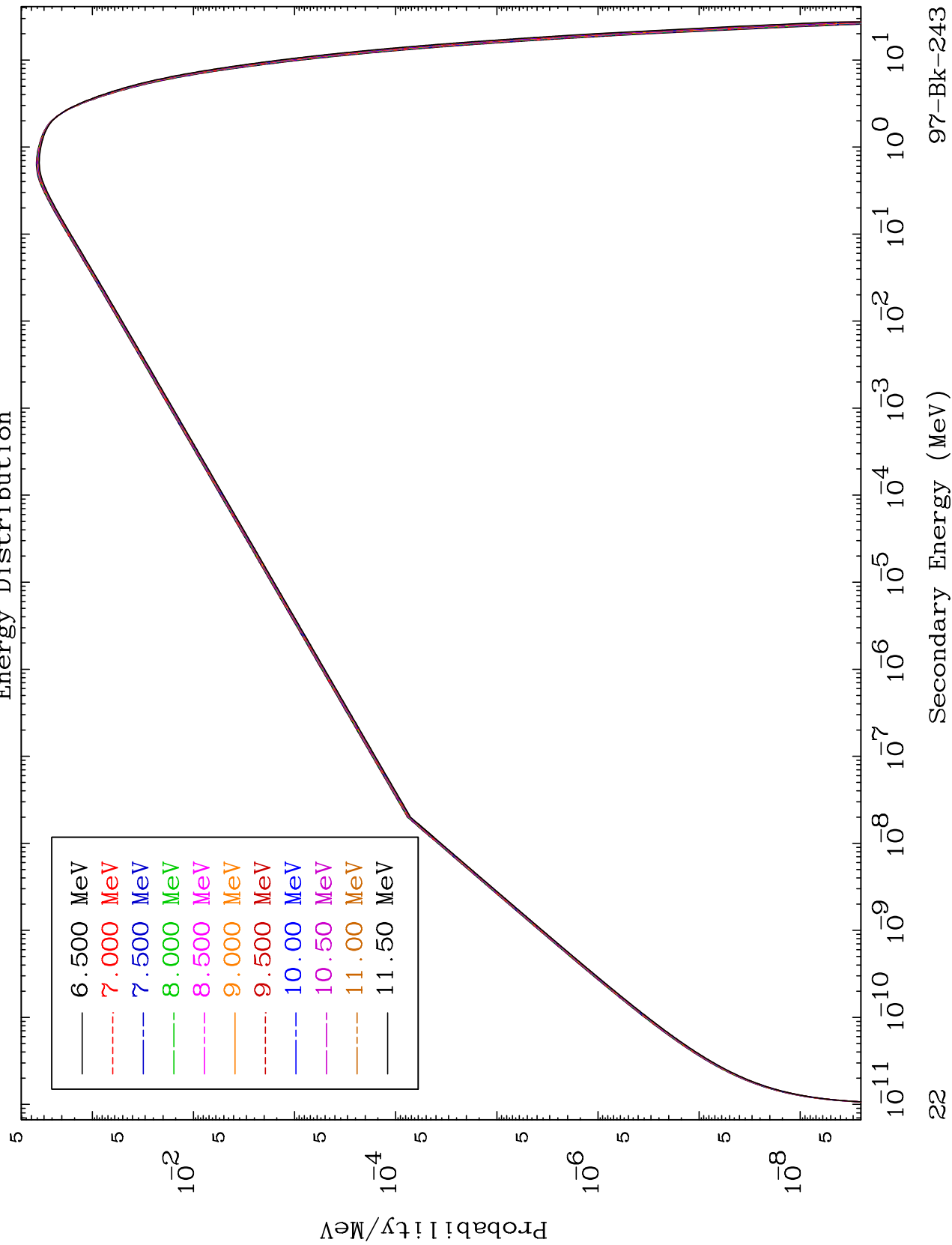
97-Bk-243



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Fission Energy Distribution

97-Bk-243



97-Bk-243

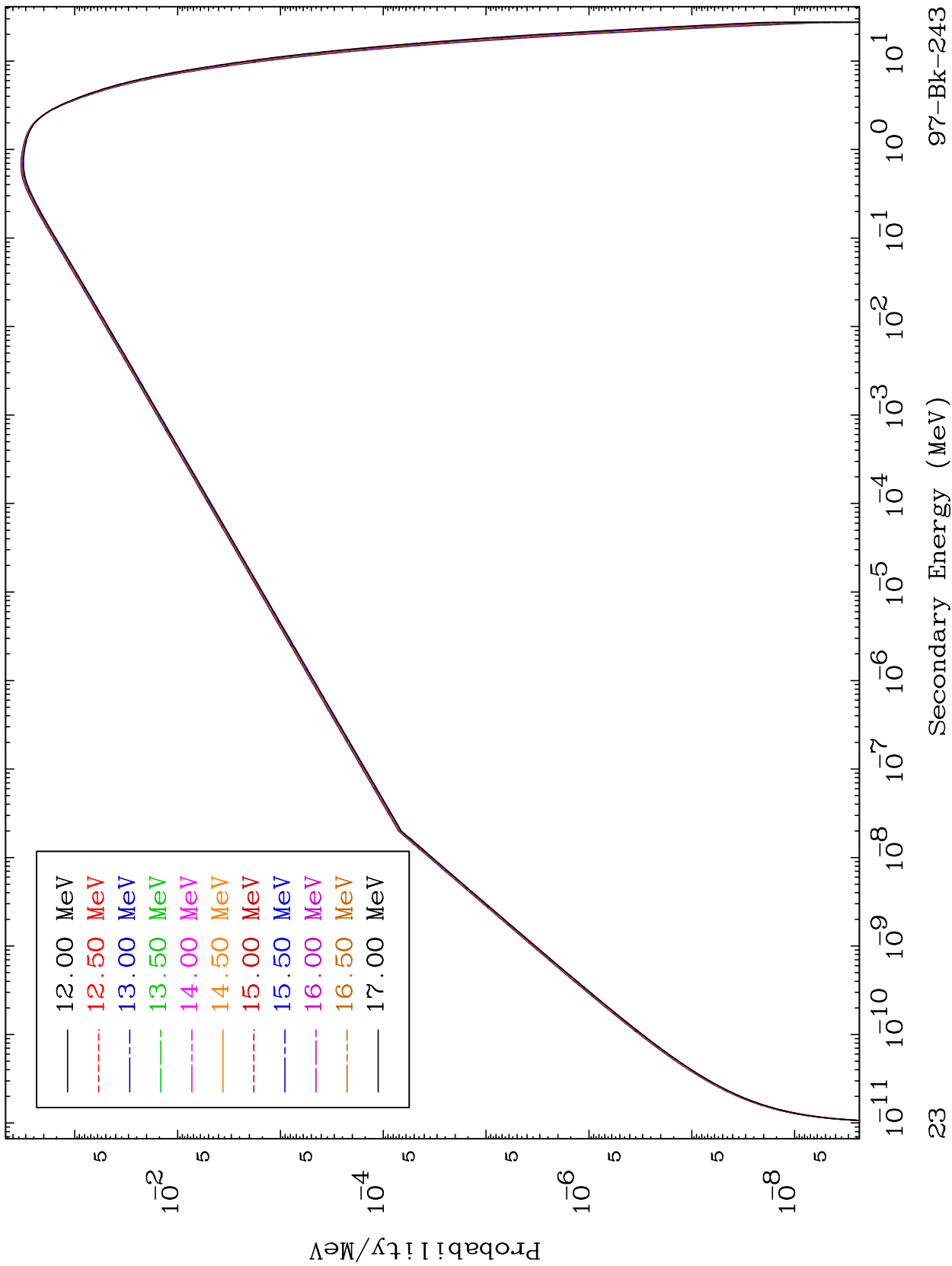
Secondary Energy (MeV)

22

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Fission Energy Distribution

97-Bk-243

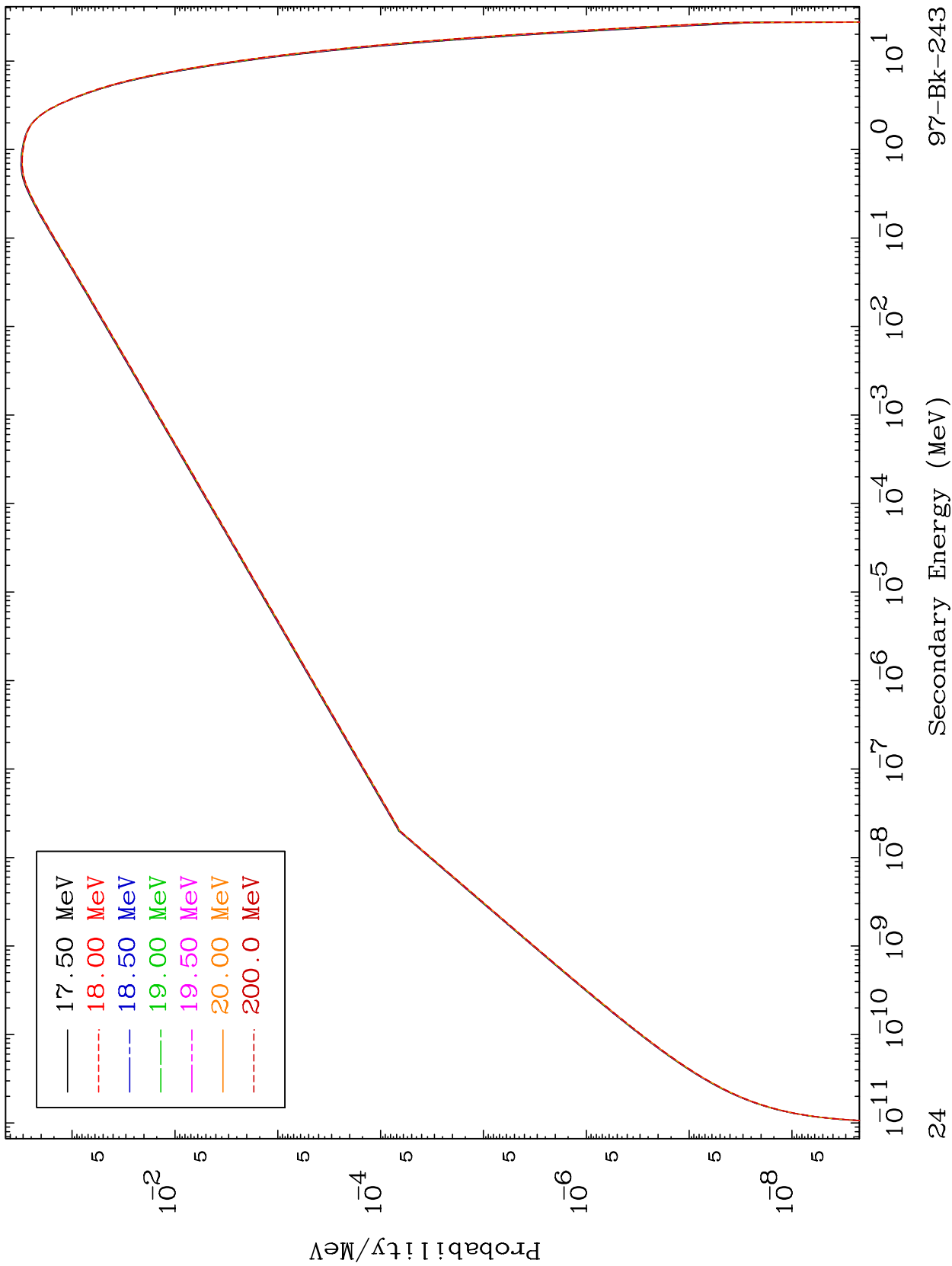


97-Bk-243

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Fission Energy Distribution

97-Bk-243



97-Bk-243

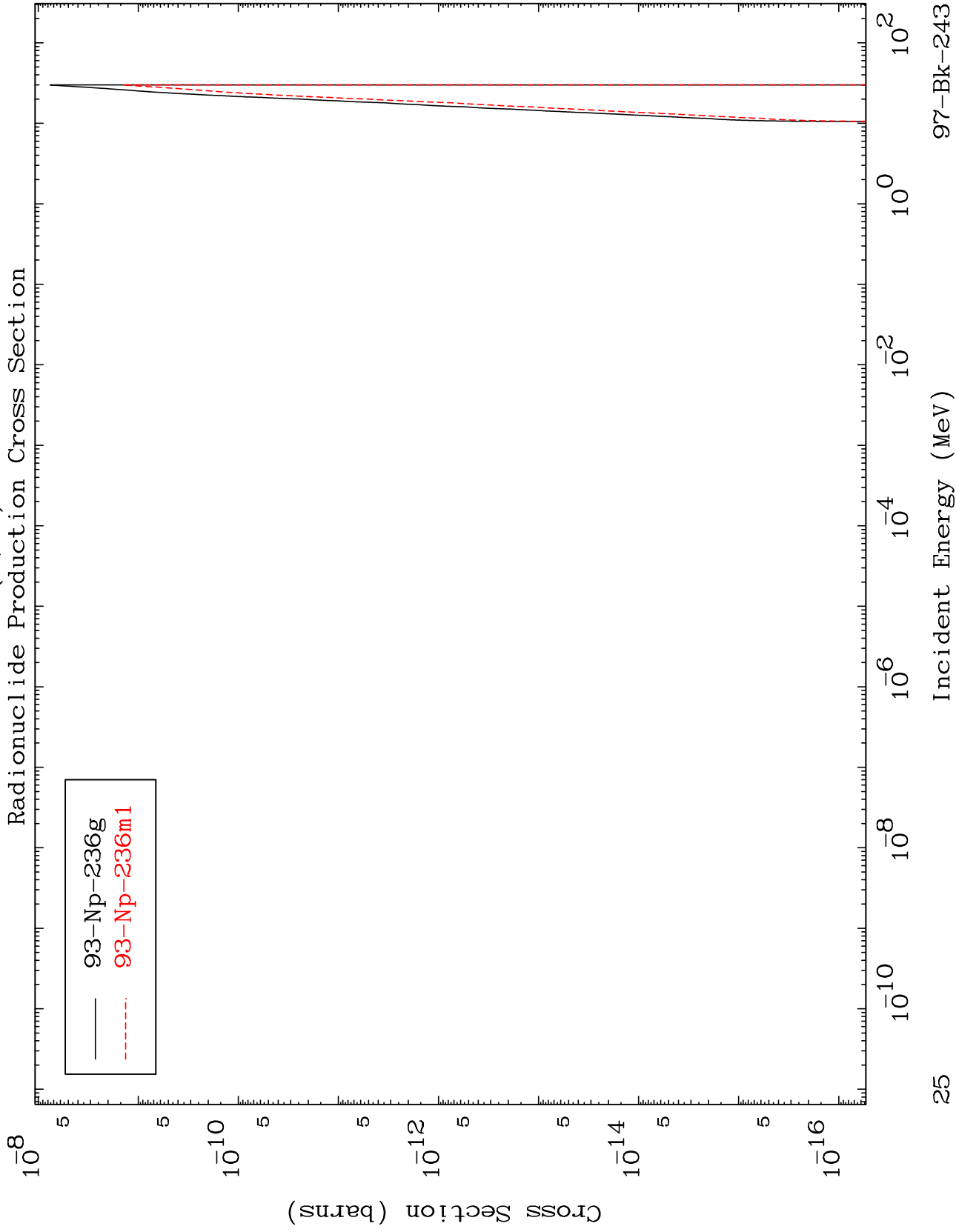
Secondary Energy (MeV)

24

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(n,2α)

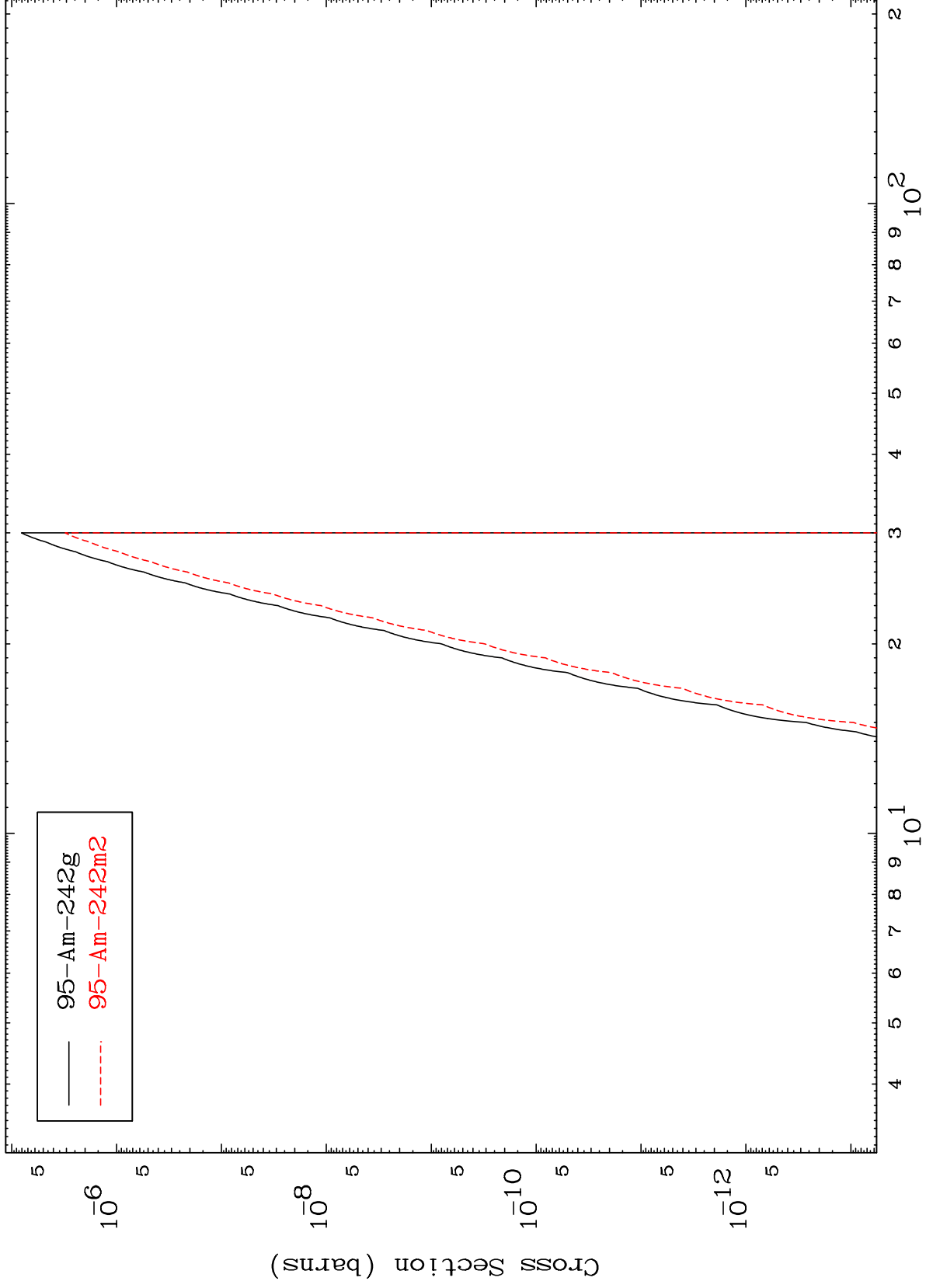
97-Bk-243



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97-Bk-243

(n,2p)
Radionuclide Production Cross Section



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Incident Energy (MeV)

97-Bk-243