

Program EVALPLOT  
(Version 2018-1)

by

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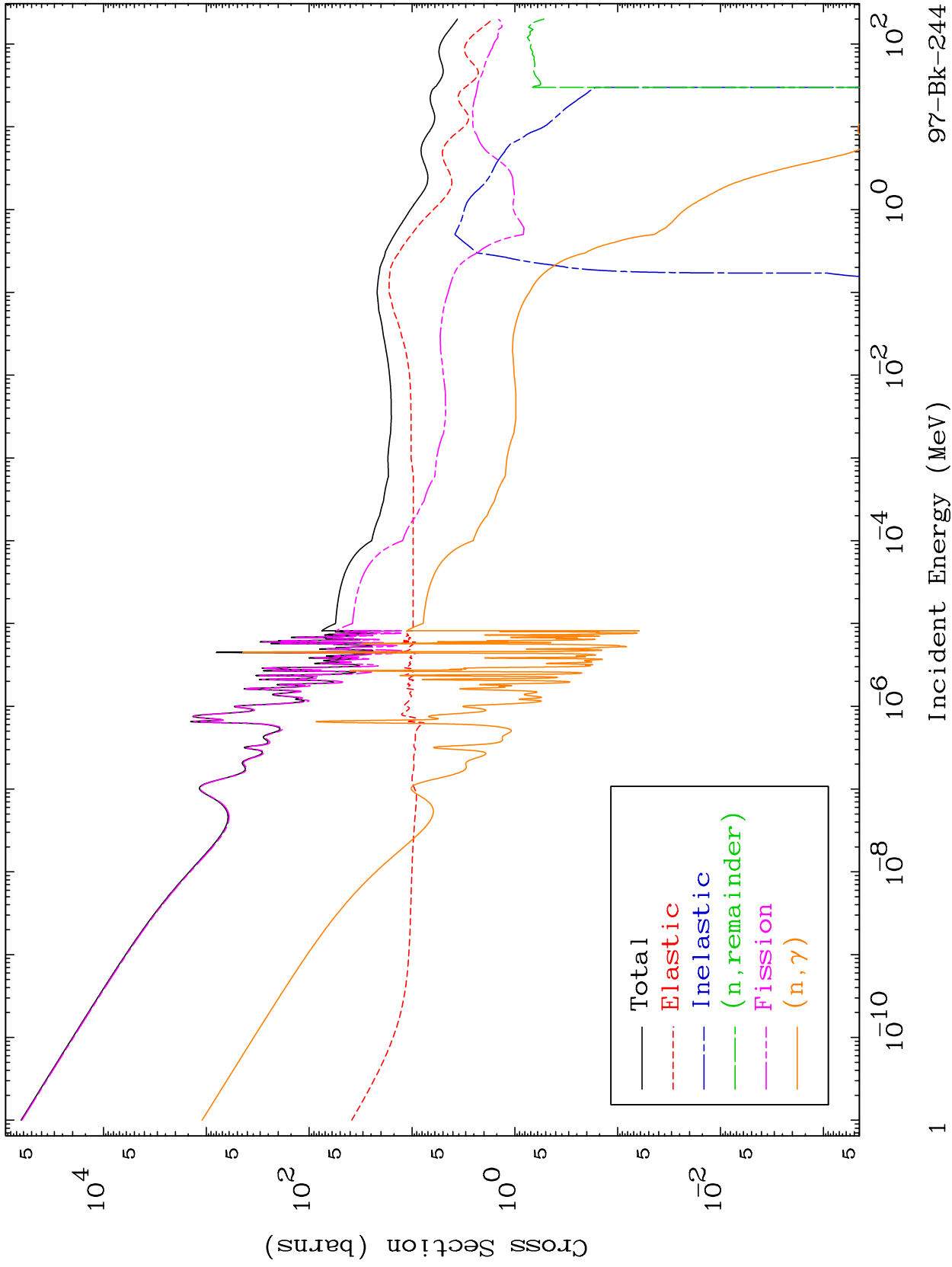
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9737

Major  
293 Kelvin Cross Sections

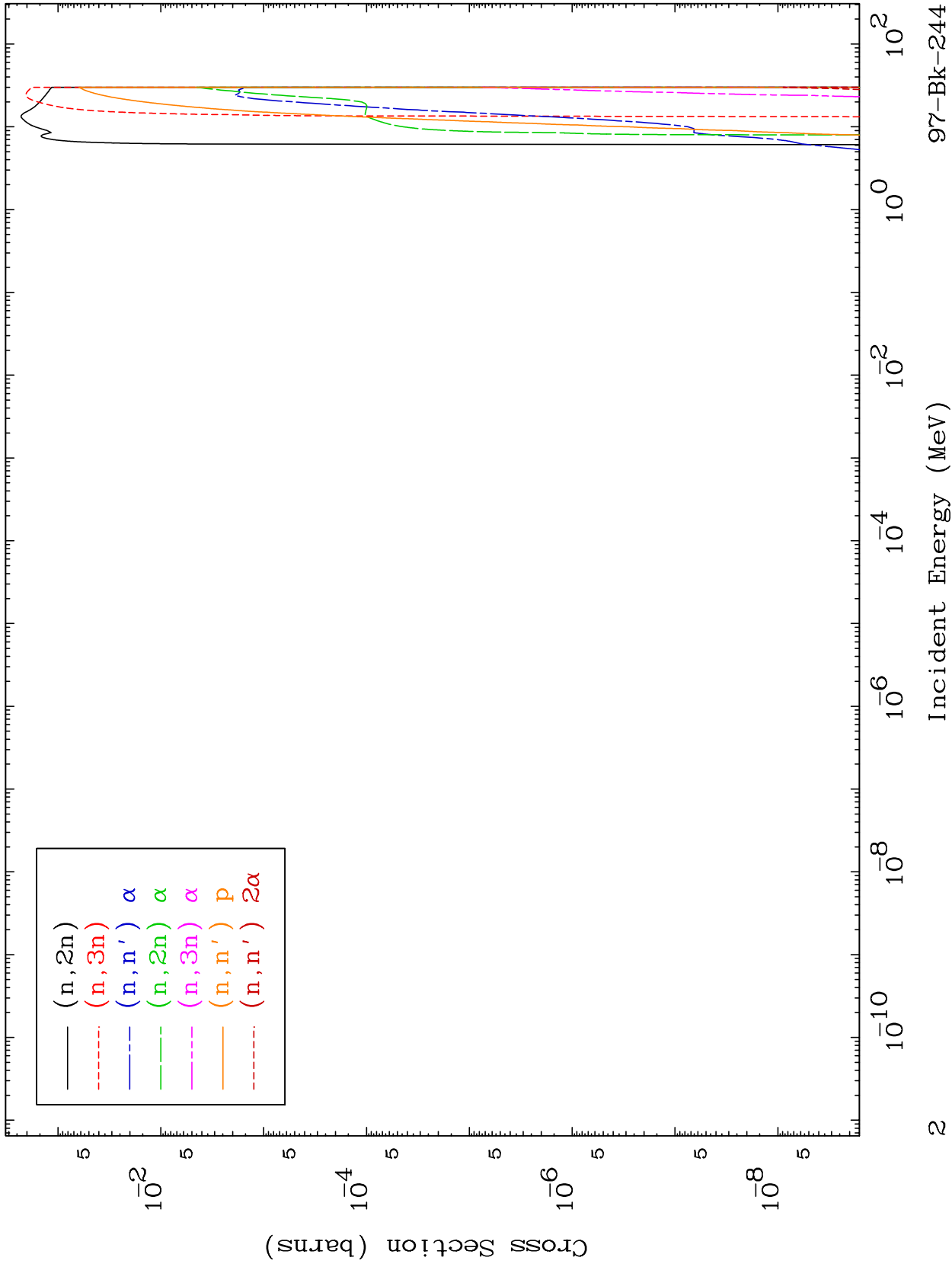
97-Bk-244

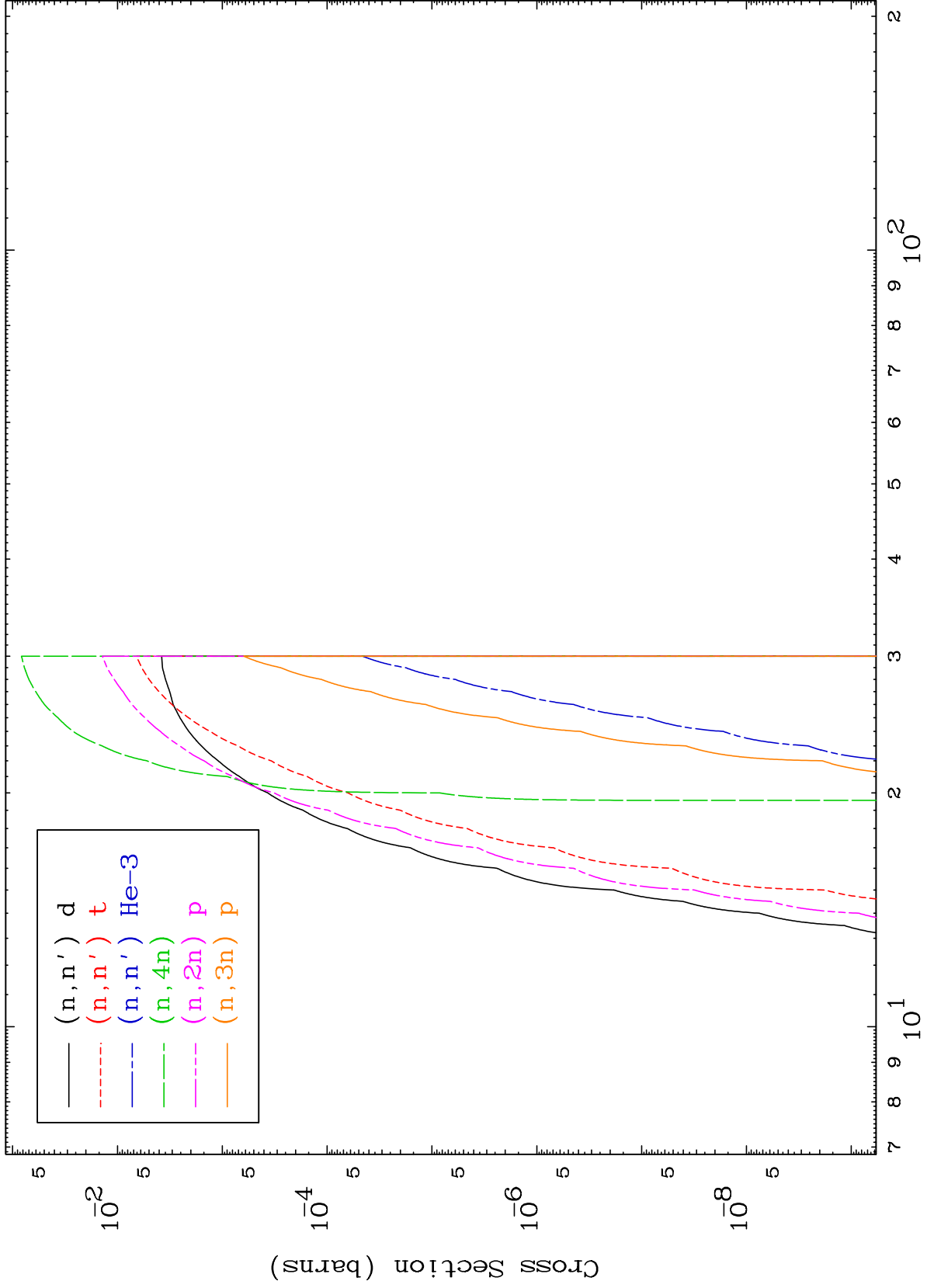


MAT 9737

Neutron Production  
293 Kelvin Cross Sections

97-Bk-244

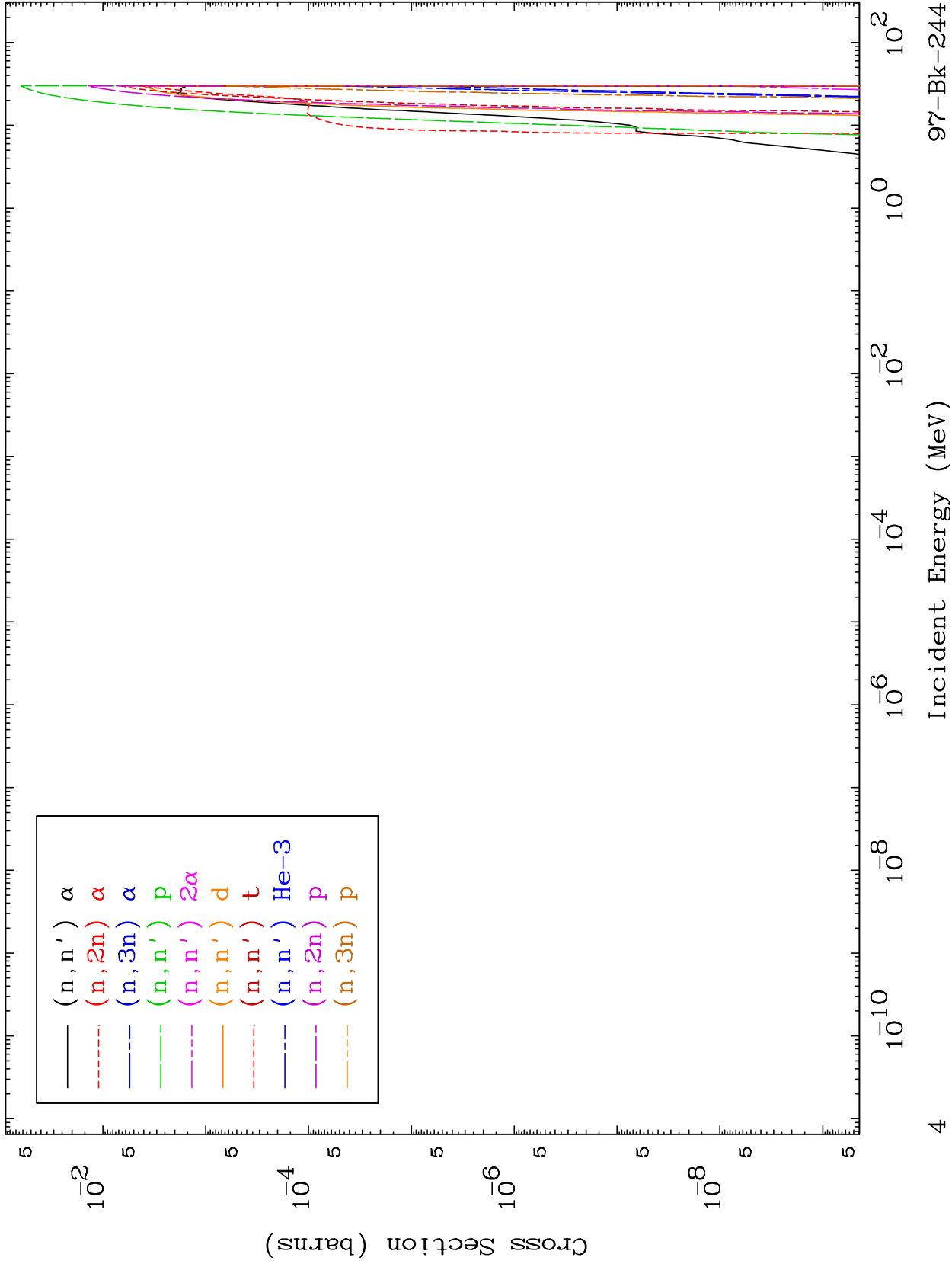


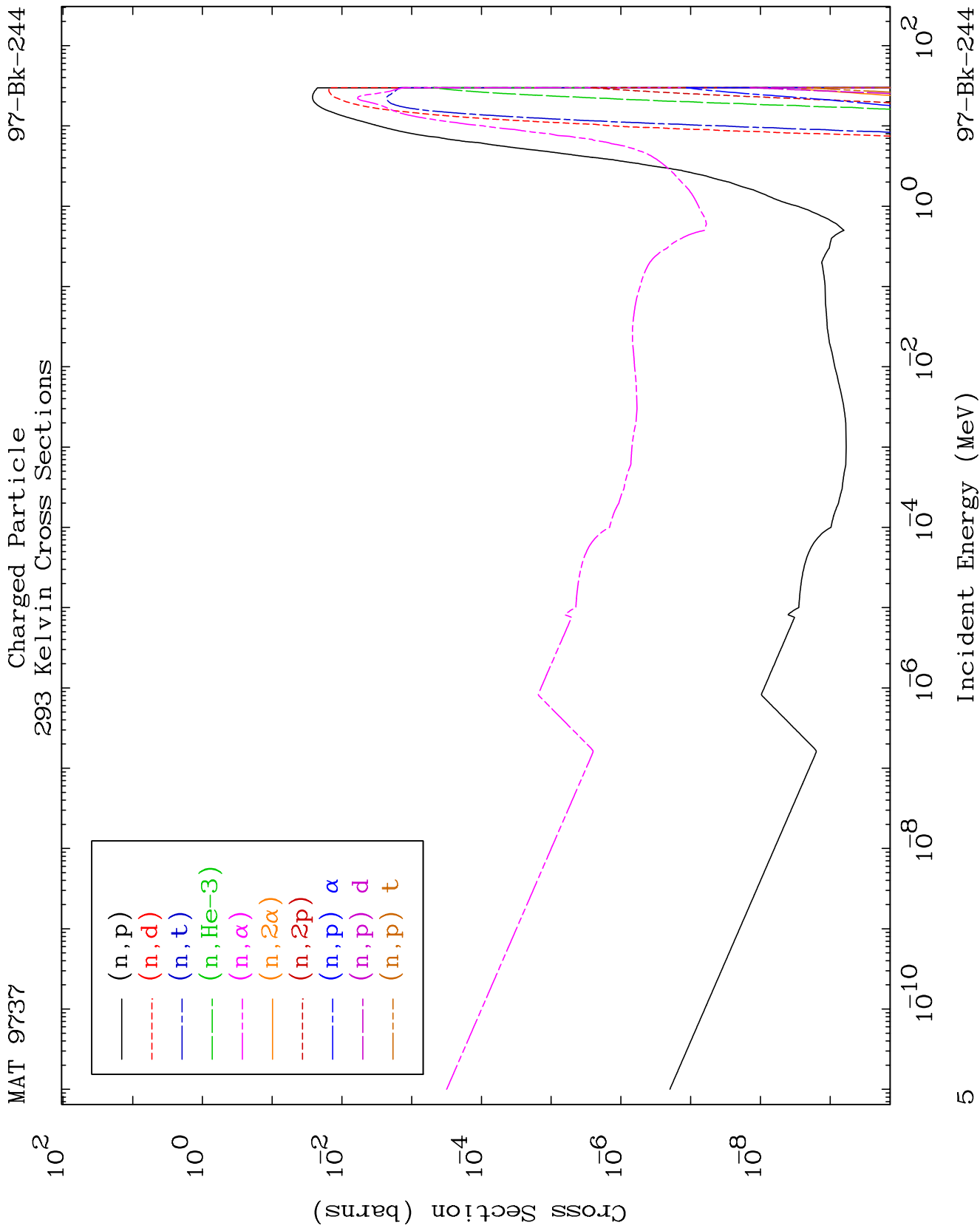


MAT 9737

Charged Particle  
293 Kelvin Cross Sections

97-Bk-244

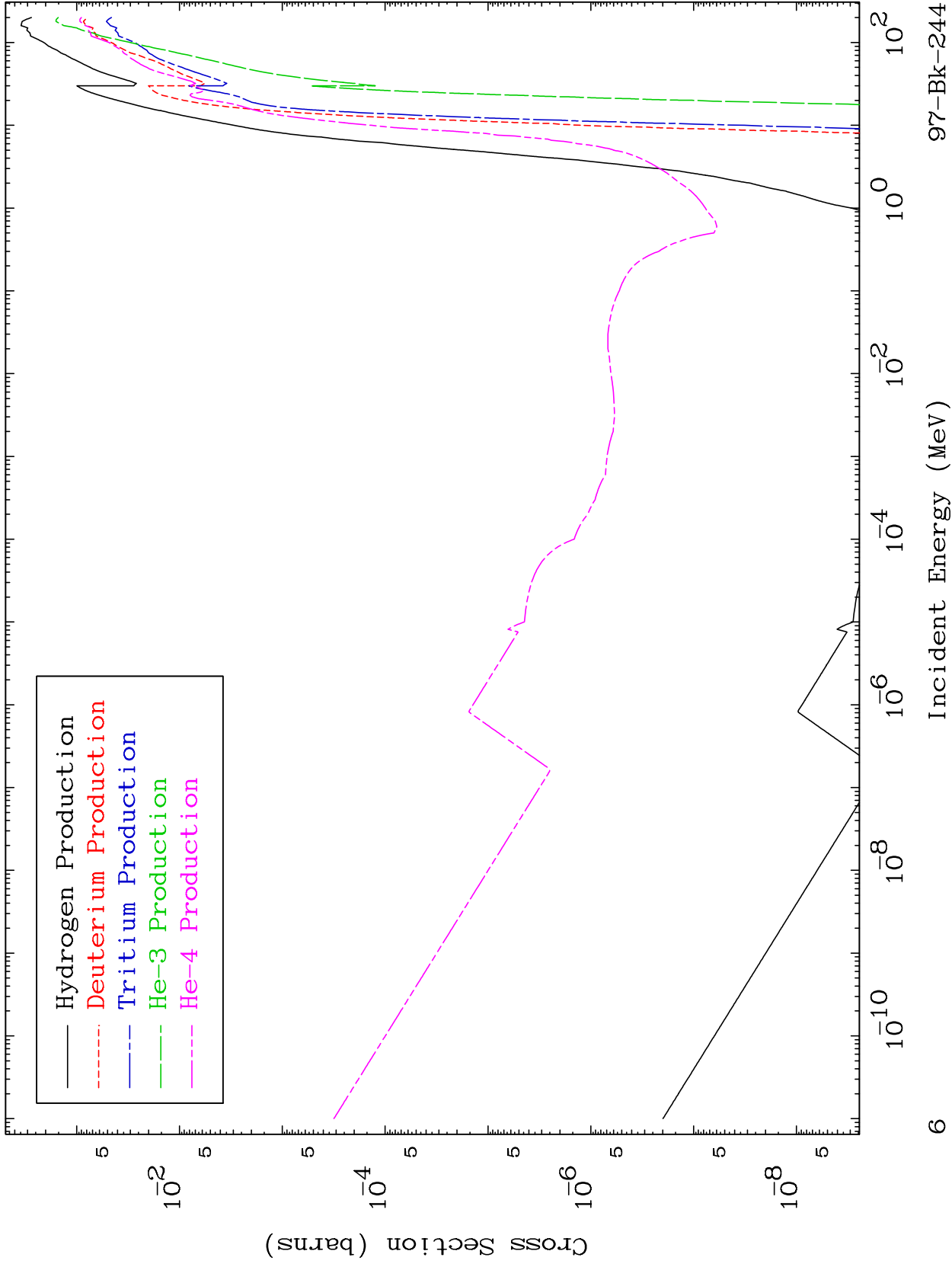




MAT 9737

Particle Production  
293 Kelvin Cross Sections

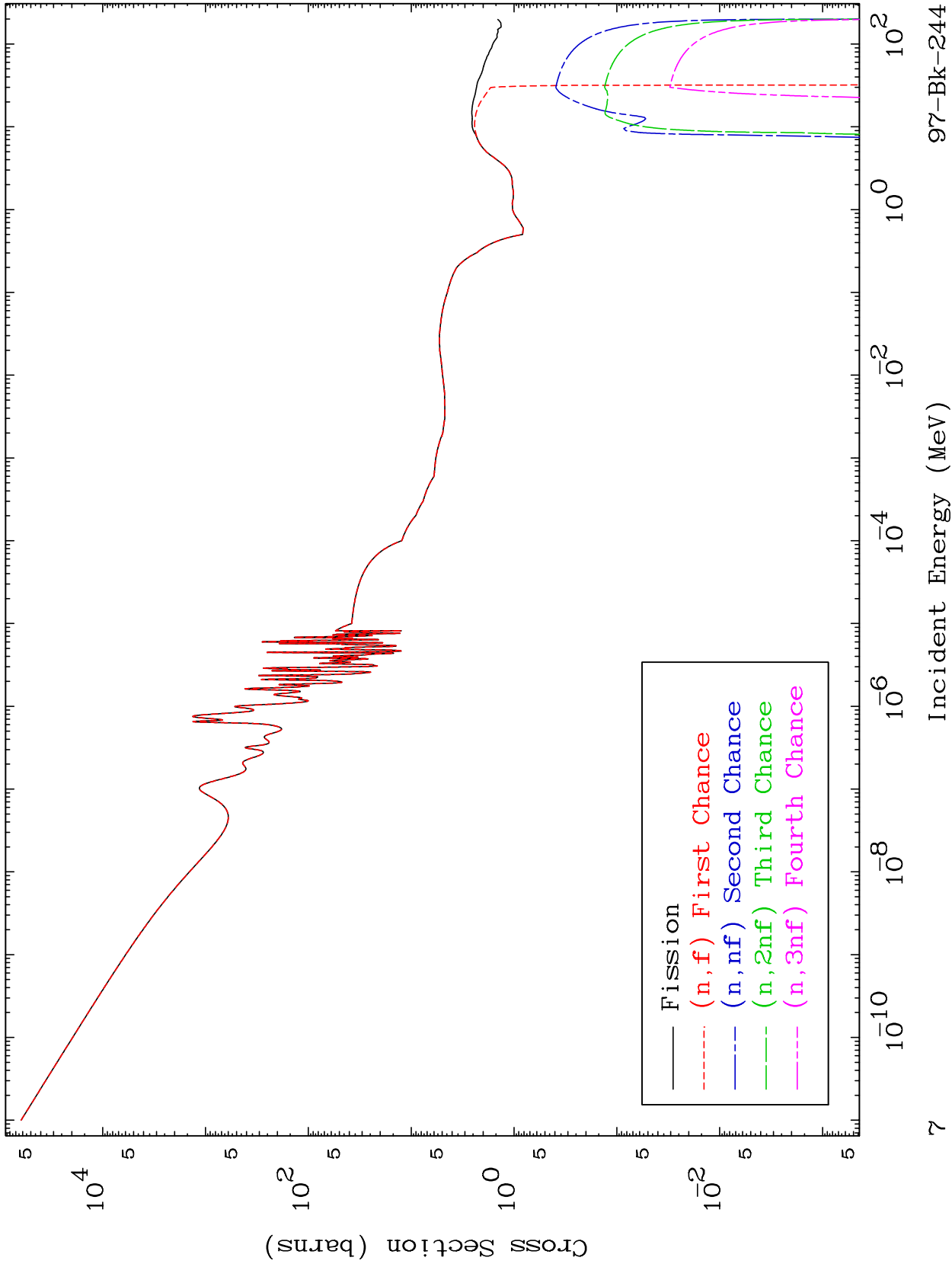
97-Bk-244



MAT 9737

Fission  
293 Kelvin Cross Sections

97-Bk-244



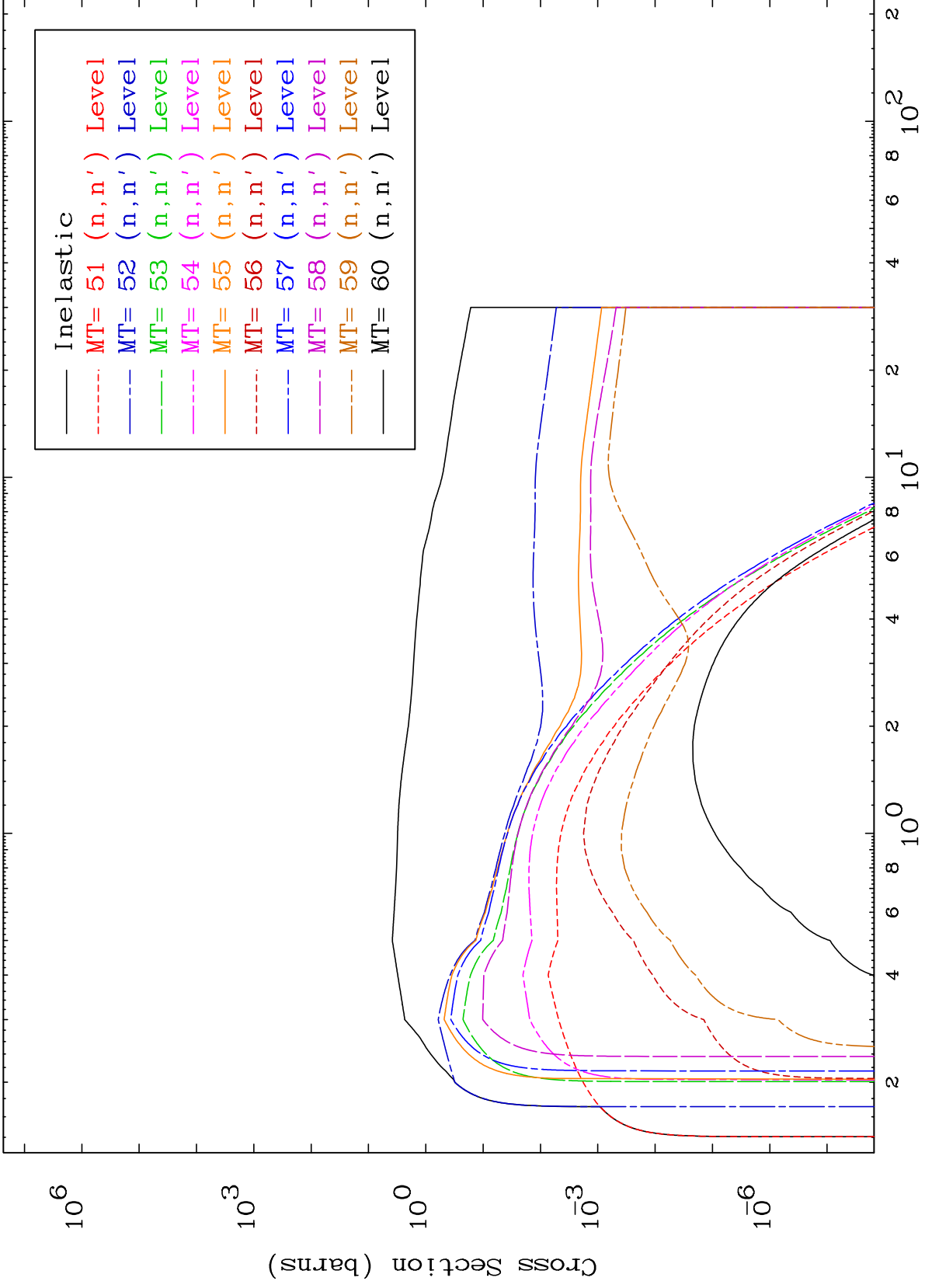


MAT 9737

(n,n') Level

97-Bk-244

293 Kelvin Cross Sections

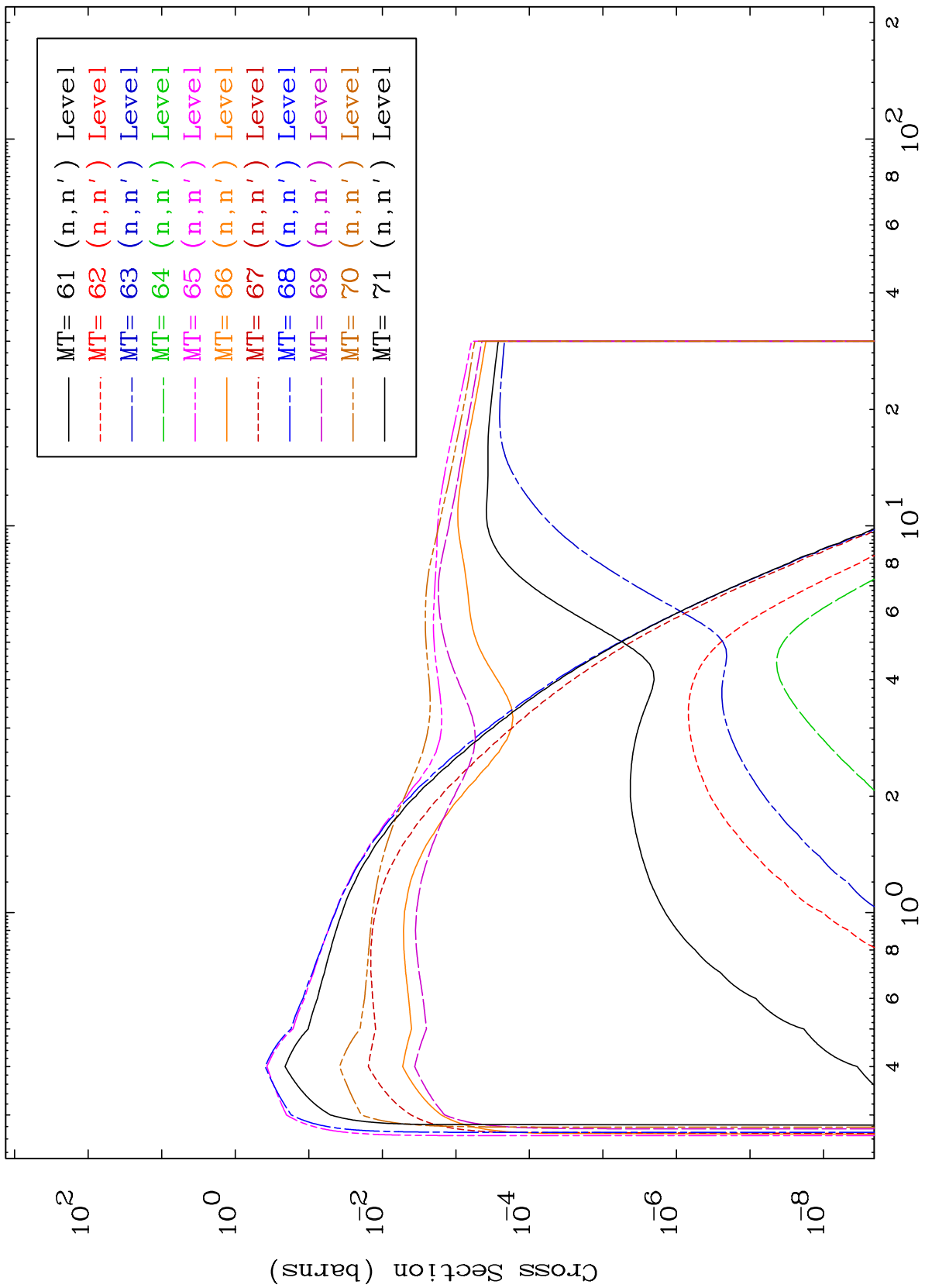


MAT 9737

(n,n') Level

293 Kelvin Cross Sections

97-Bk-244

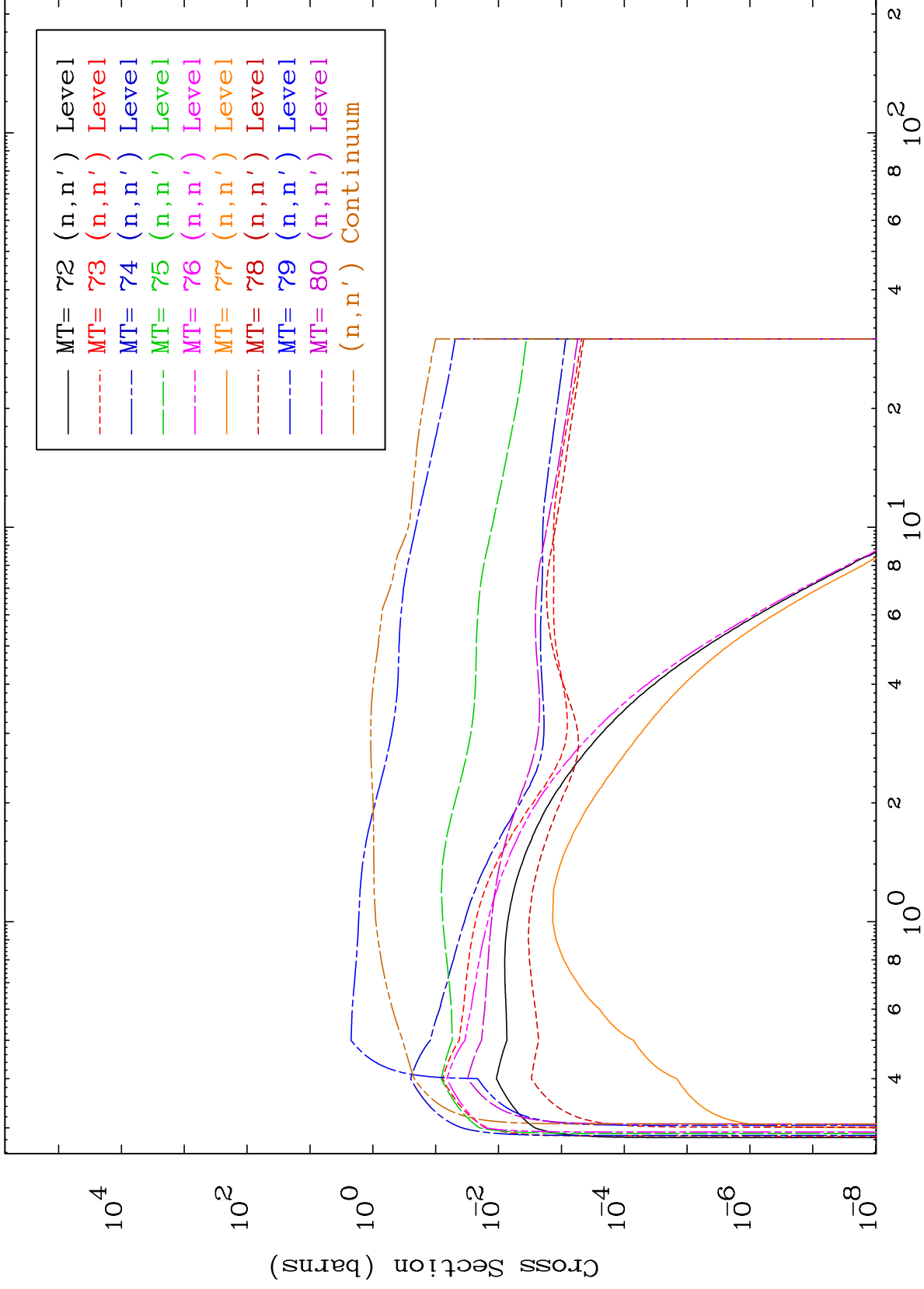


MAT 9737

(n,n') Level

97-Bk-244

293 Kelvin Cross Sections



10

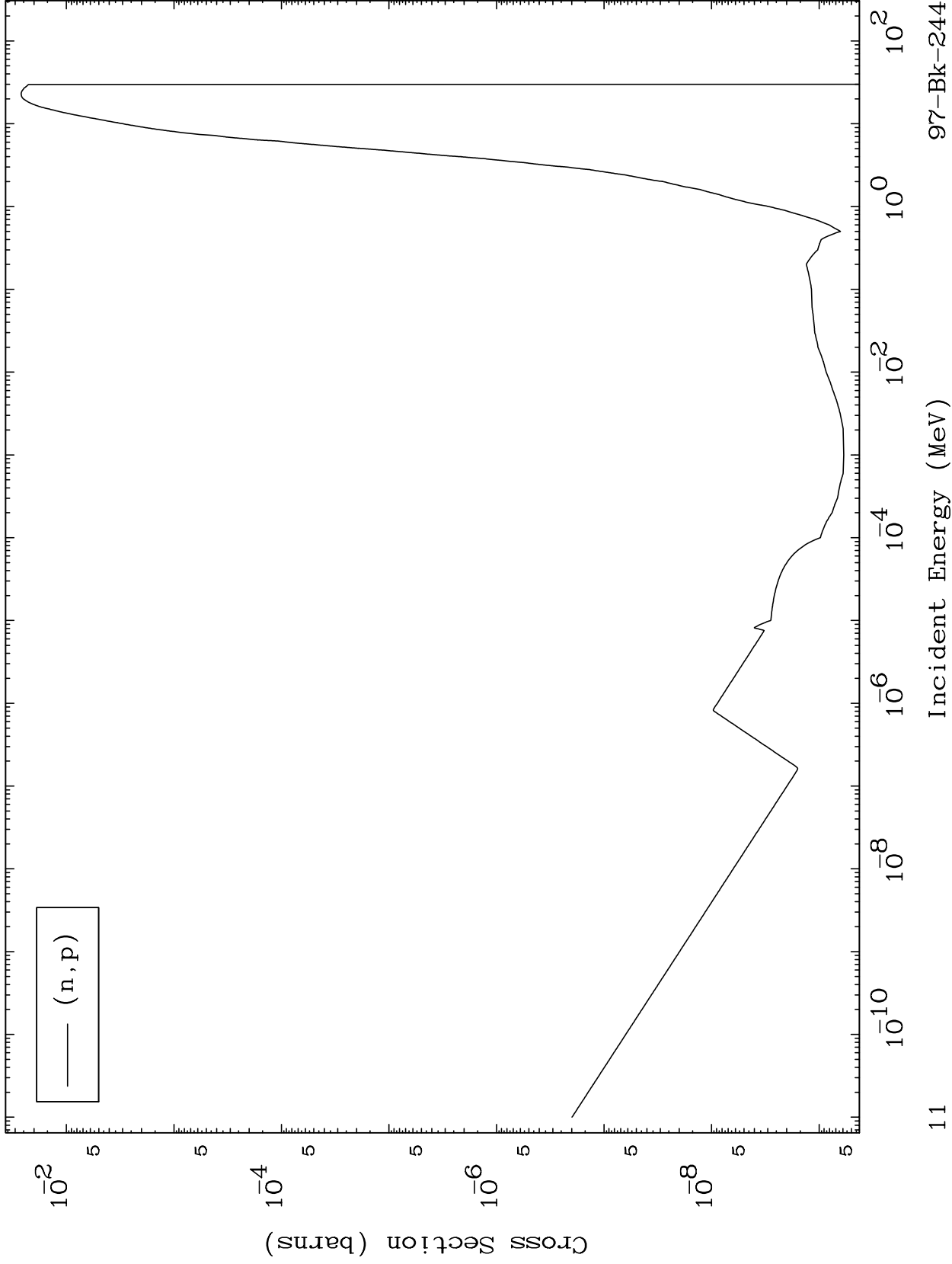
Incident Energy (MeV)

97-Bk-244

MAT 9737

(n,p) Levels  
293 Kelvin Cross Sections

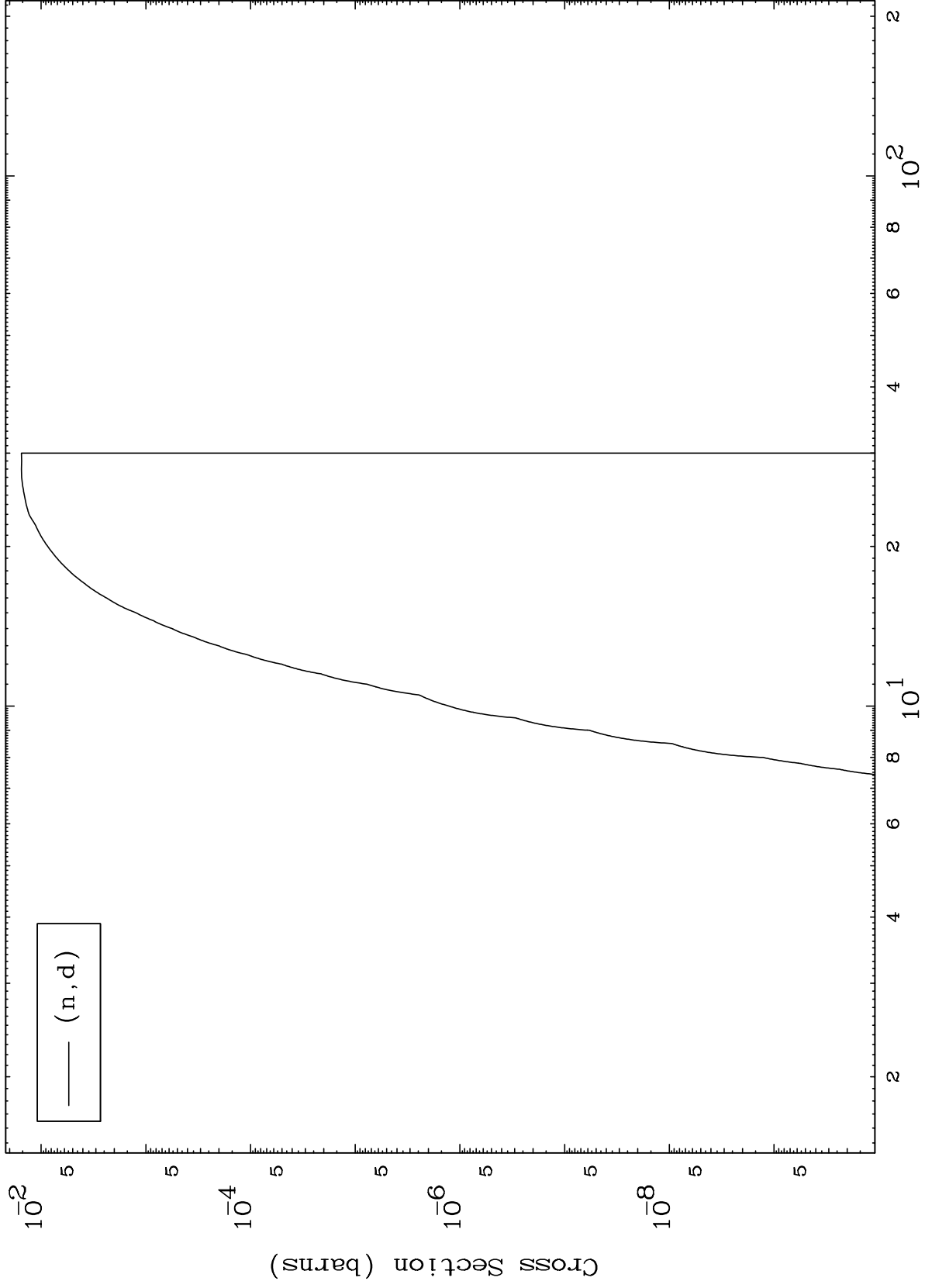
97-Bk-244



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-244



12

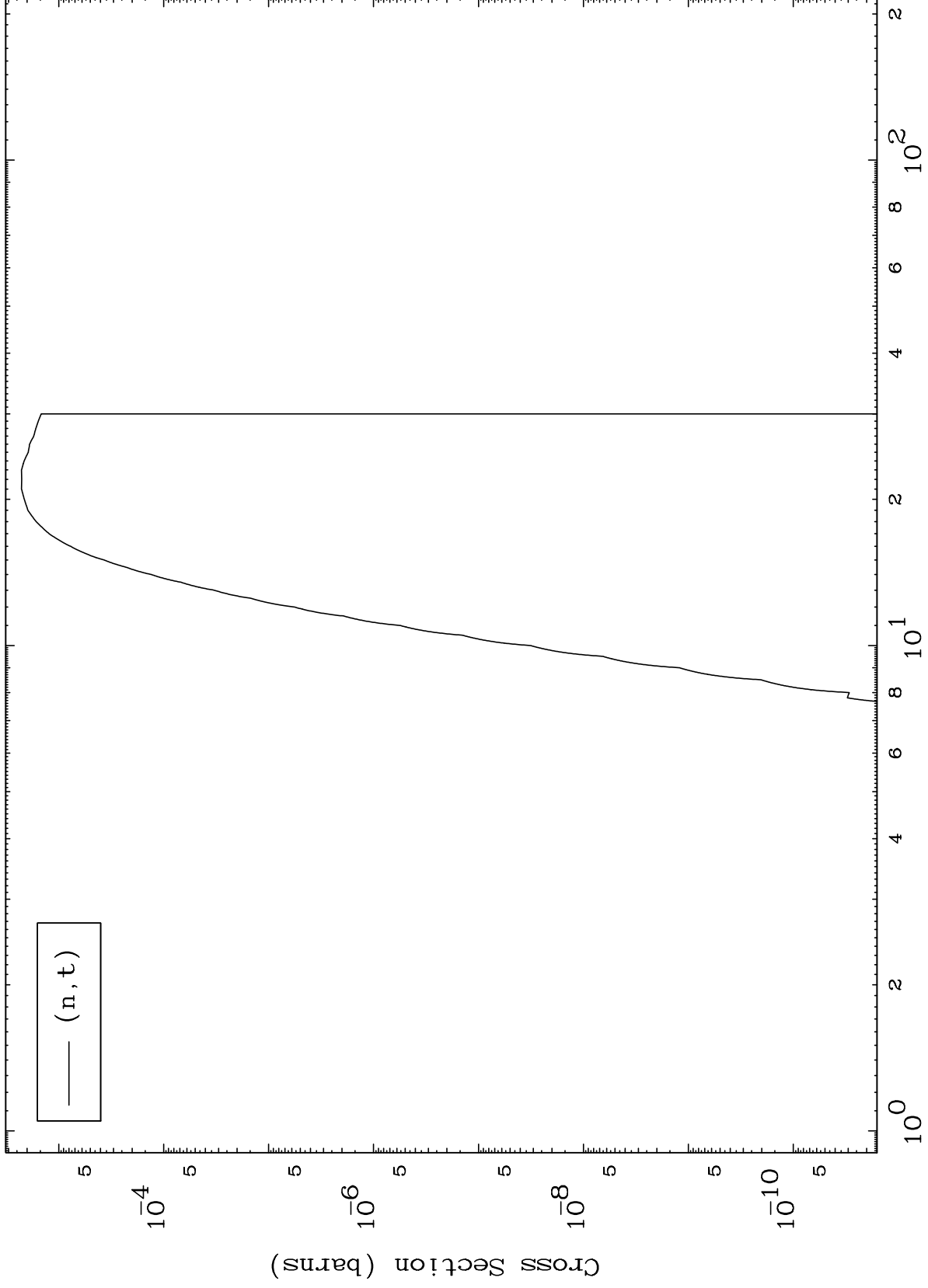
Incident Energy (MeV)

97-Bk-244

MAT 9737

(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-244



13

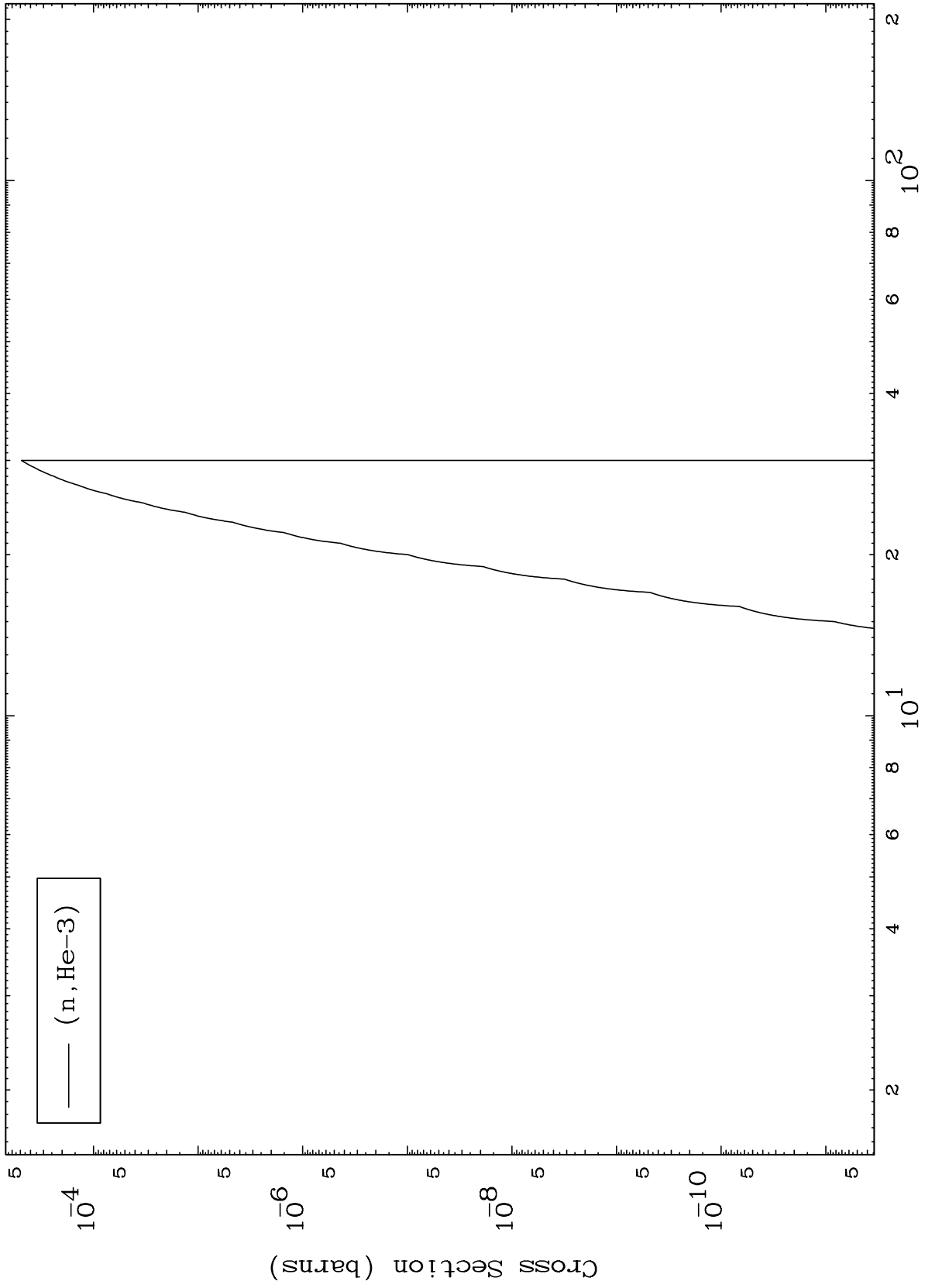
Incident Energy (MeV)

97-Bk-244

MAT 9737

(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-244



14

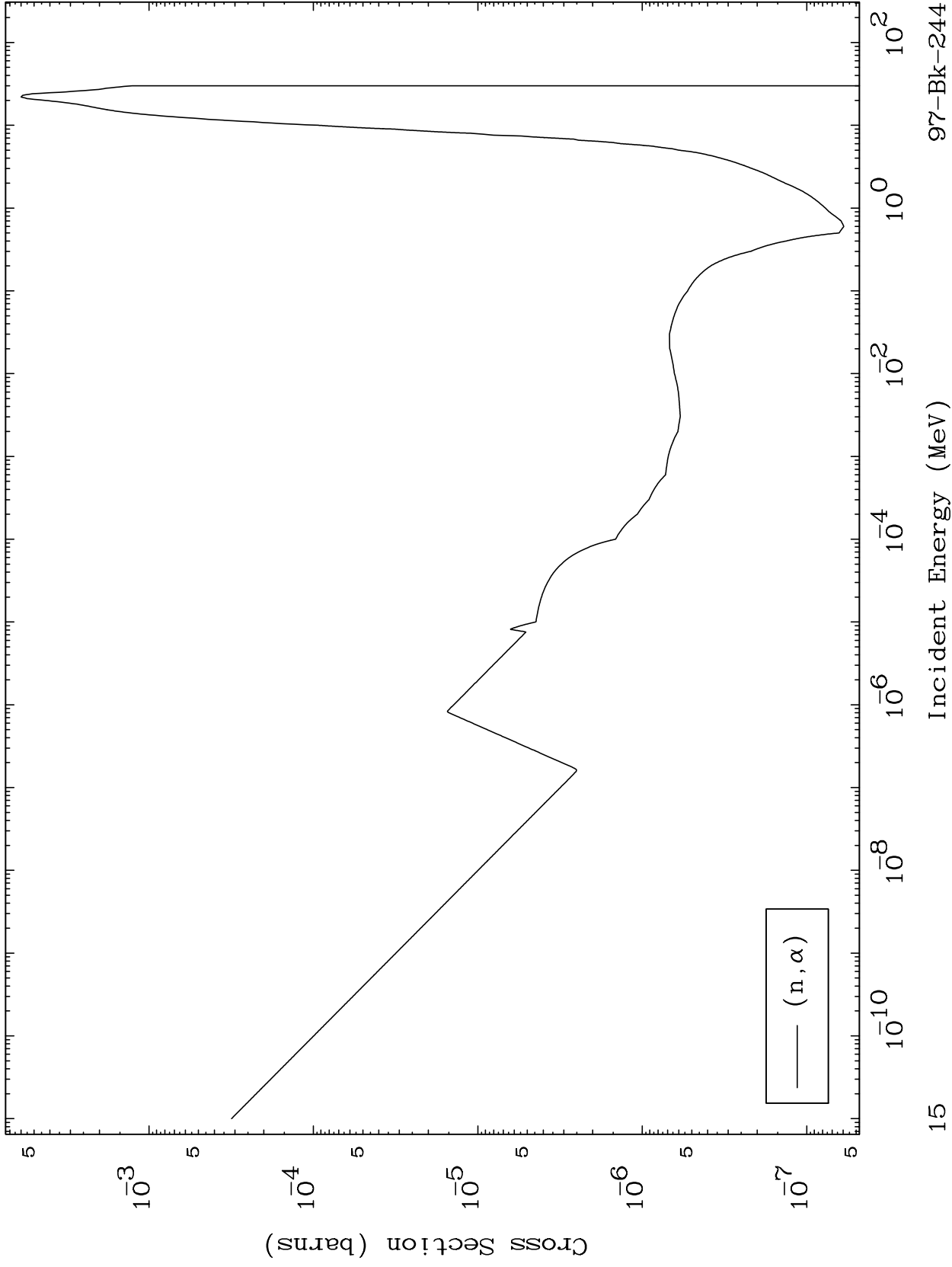
Incident Energy (MeV)

97-Bk-244

MAT 9737

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

97-Bk-244

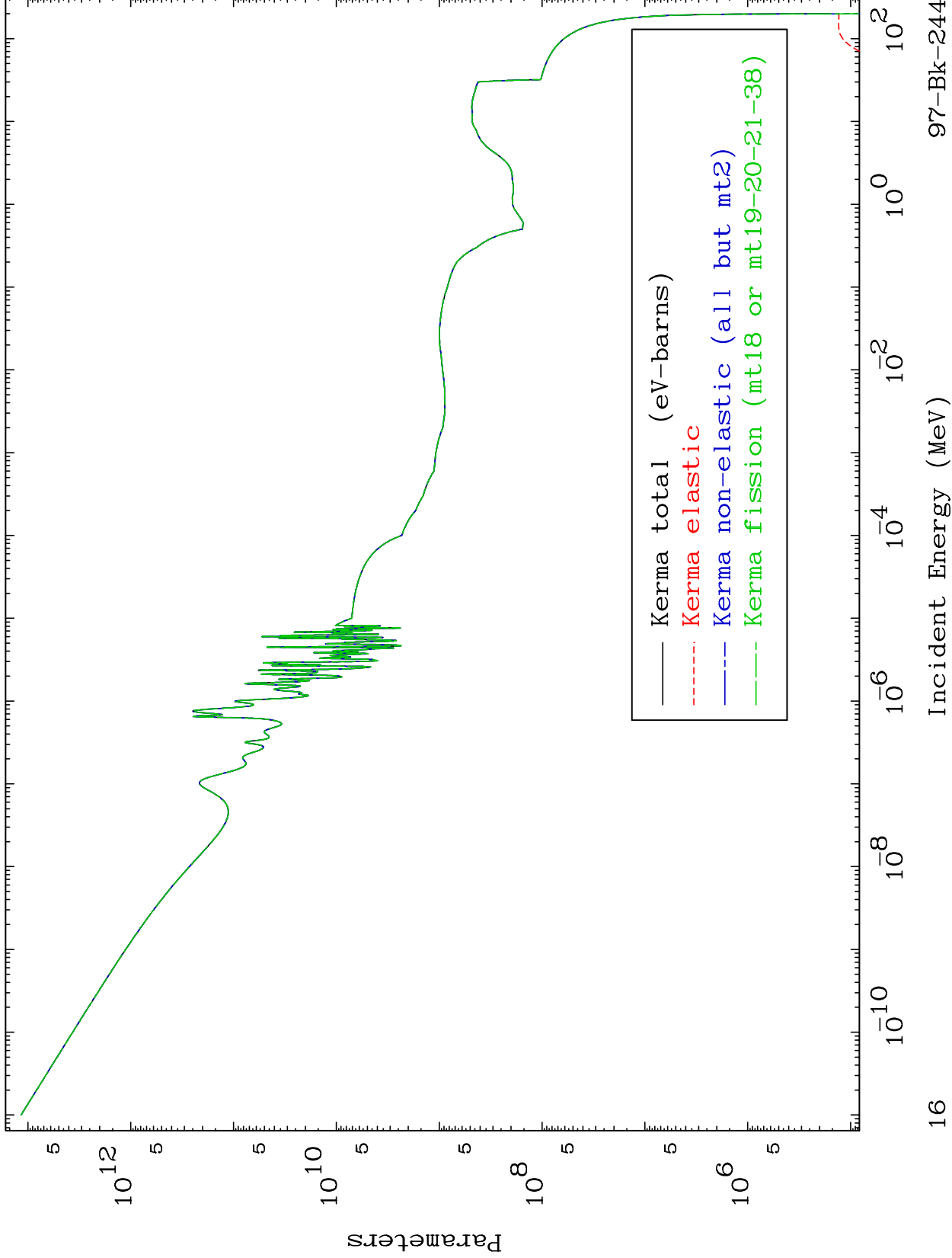


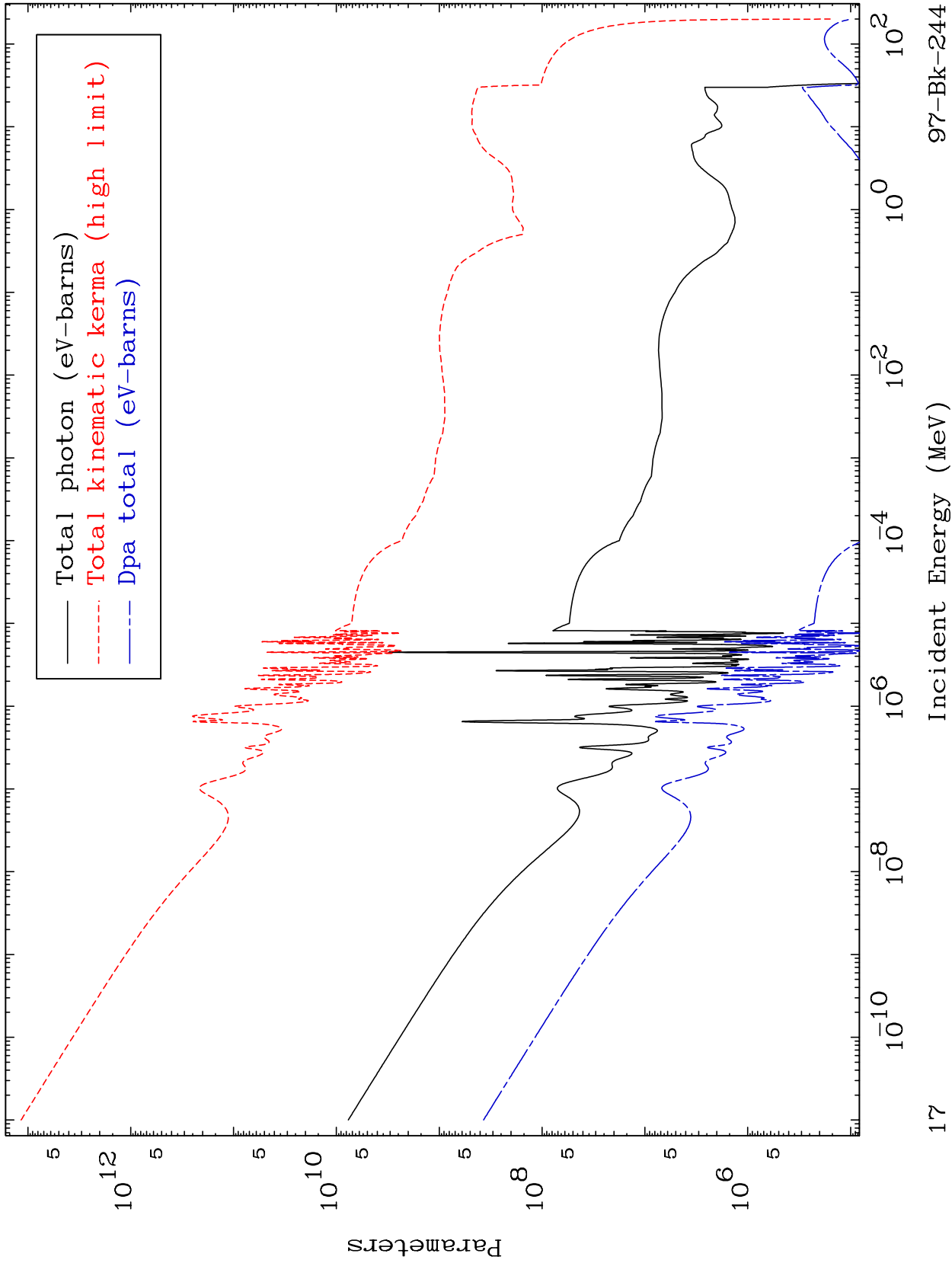
15

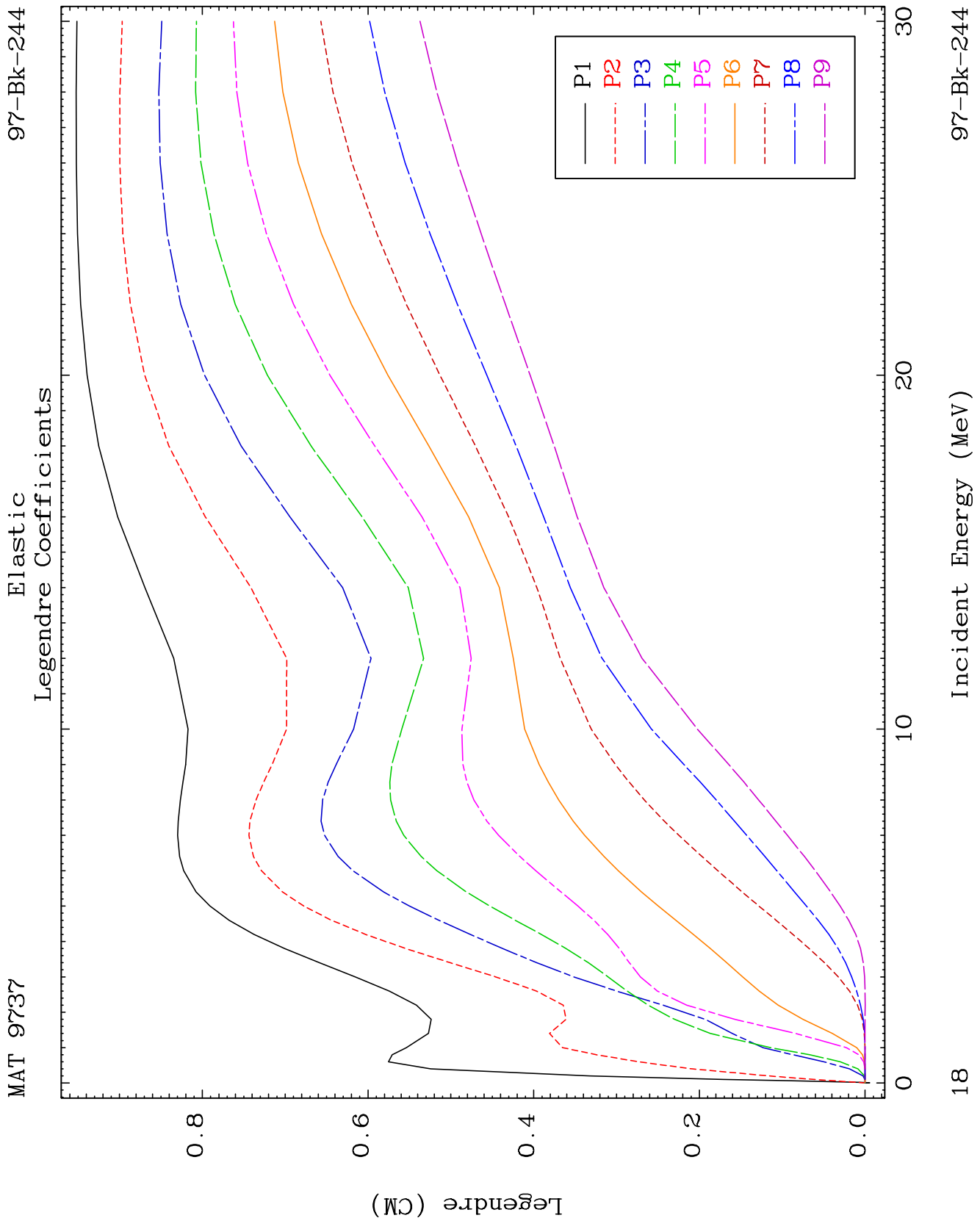
Incident Energy (MeV)

97-Bk-244









97-Bk-244

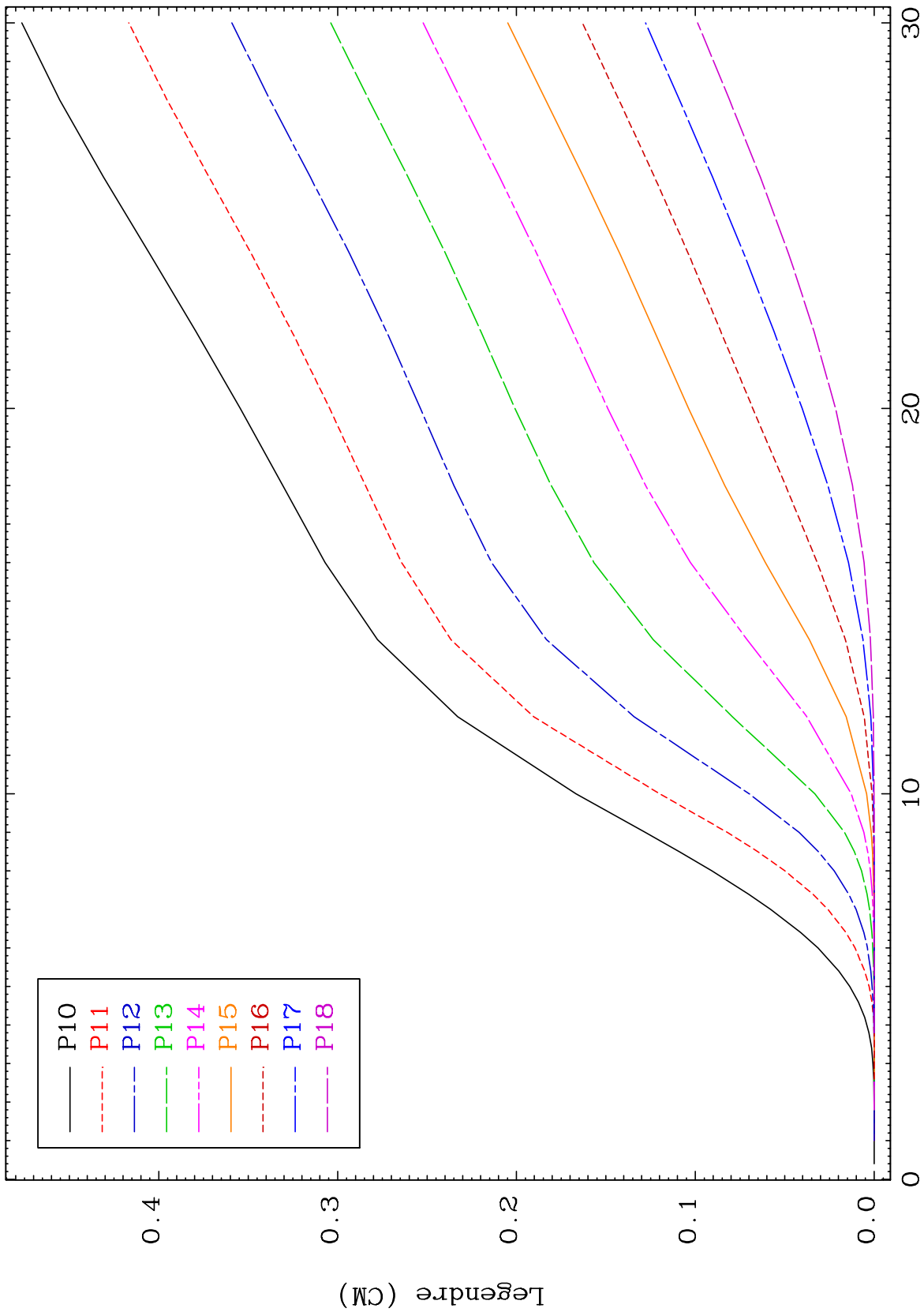
Incident Energy (MeV)

18

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Elastic Legendre Coefficients

97-Bk-244



19

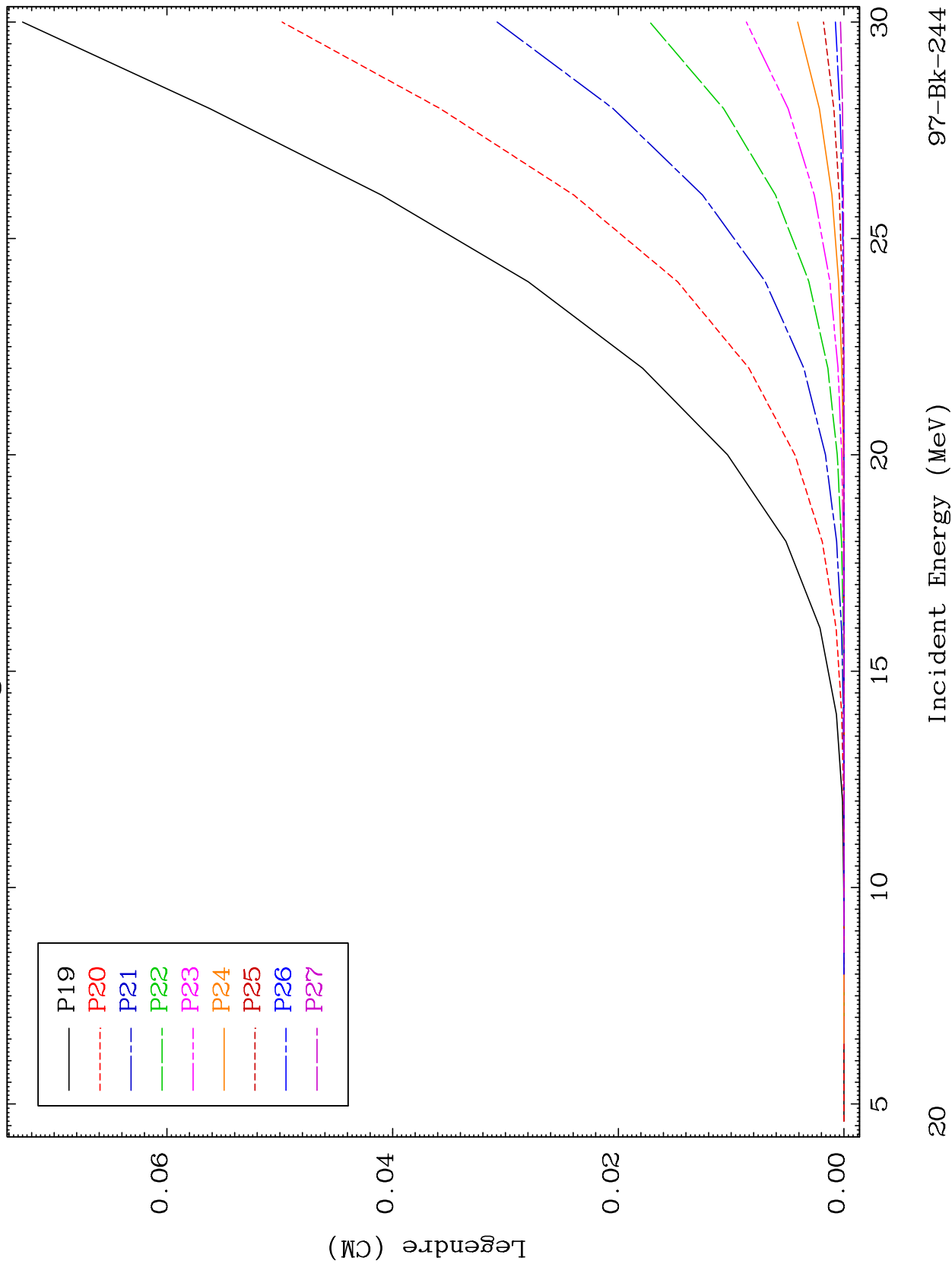
Incident Energy (MeV)

97-Bk-244

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### Elastic Legendre Coefficients

97-Bk-244



97-Bk-244

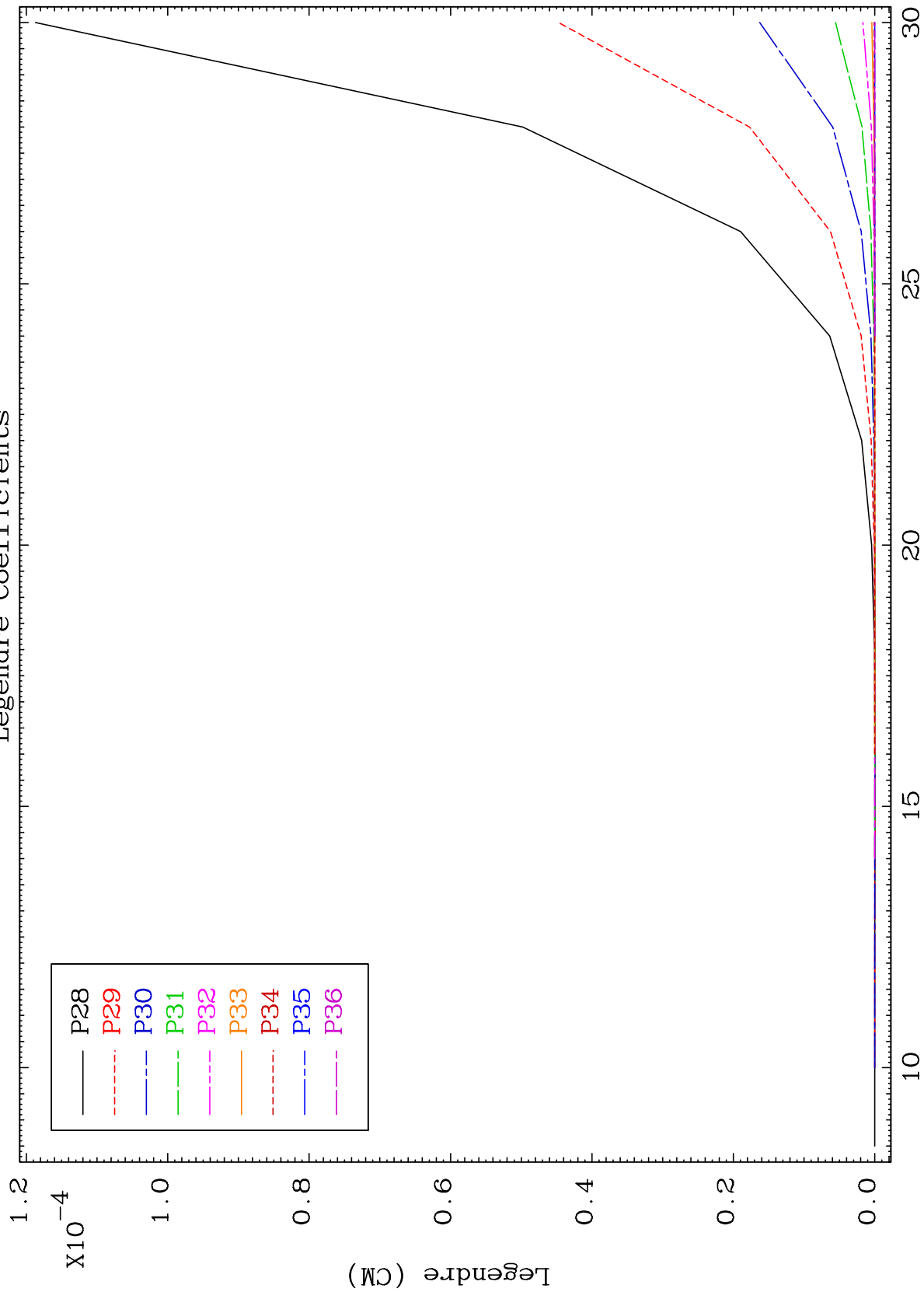
Incident Energy (MeV)

20

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Elastic  
Legendre Coefficients

97-Bk-244



21

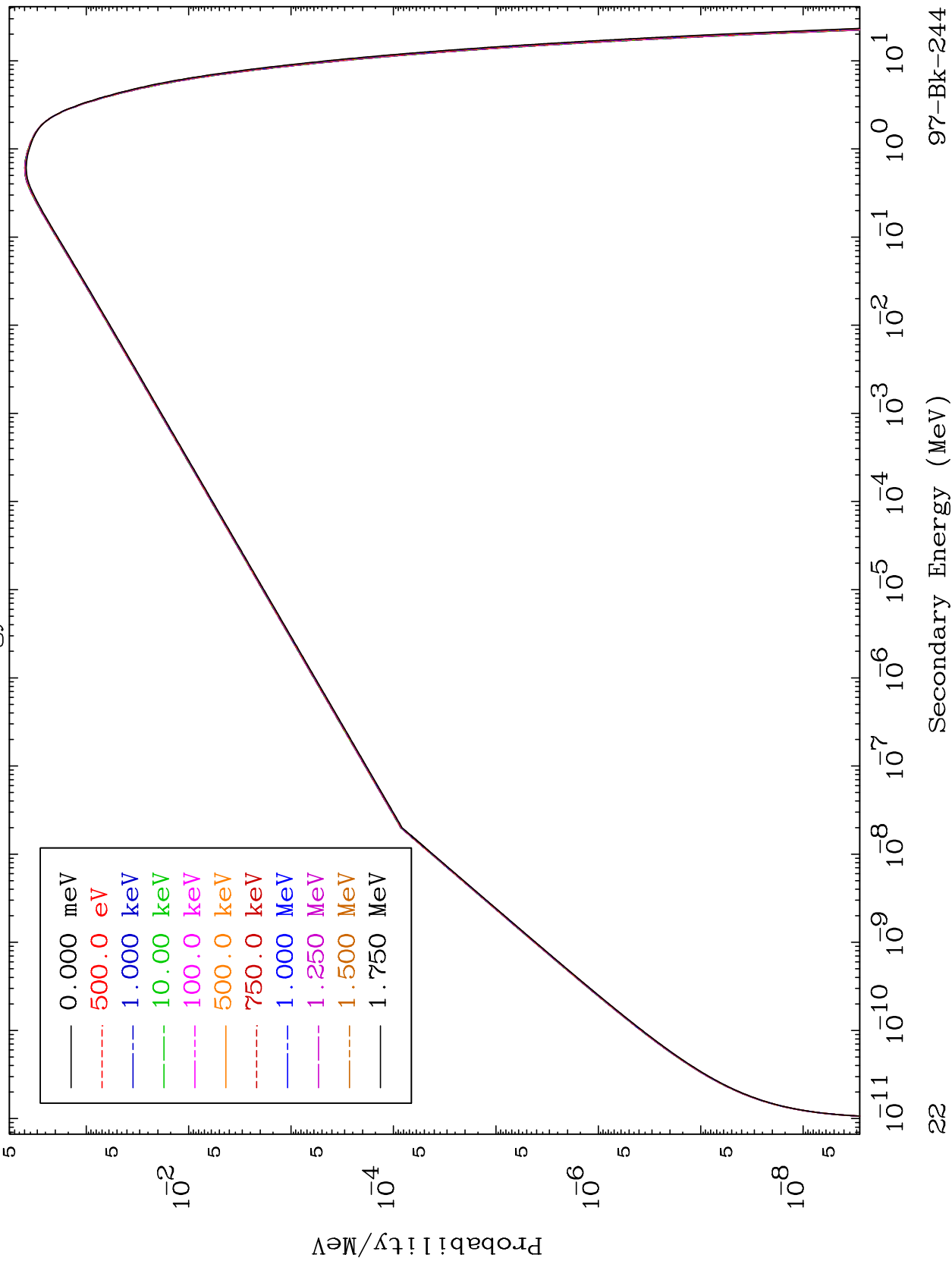
Incident Energy (MeV)

97-Bk-244

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### Fission Energy Distribution

97-Bk-244



97-Bk-244

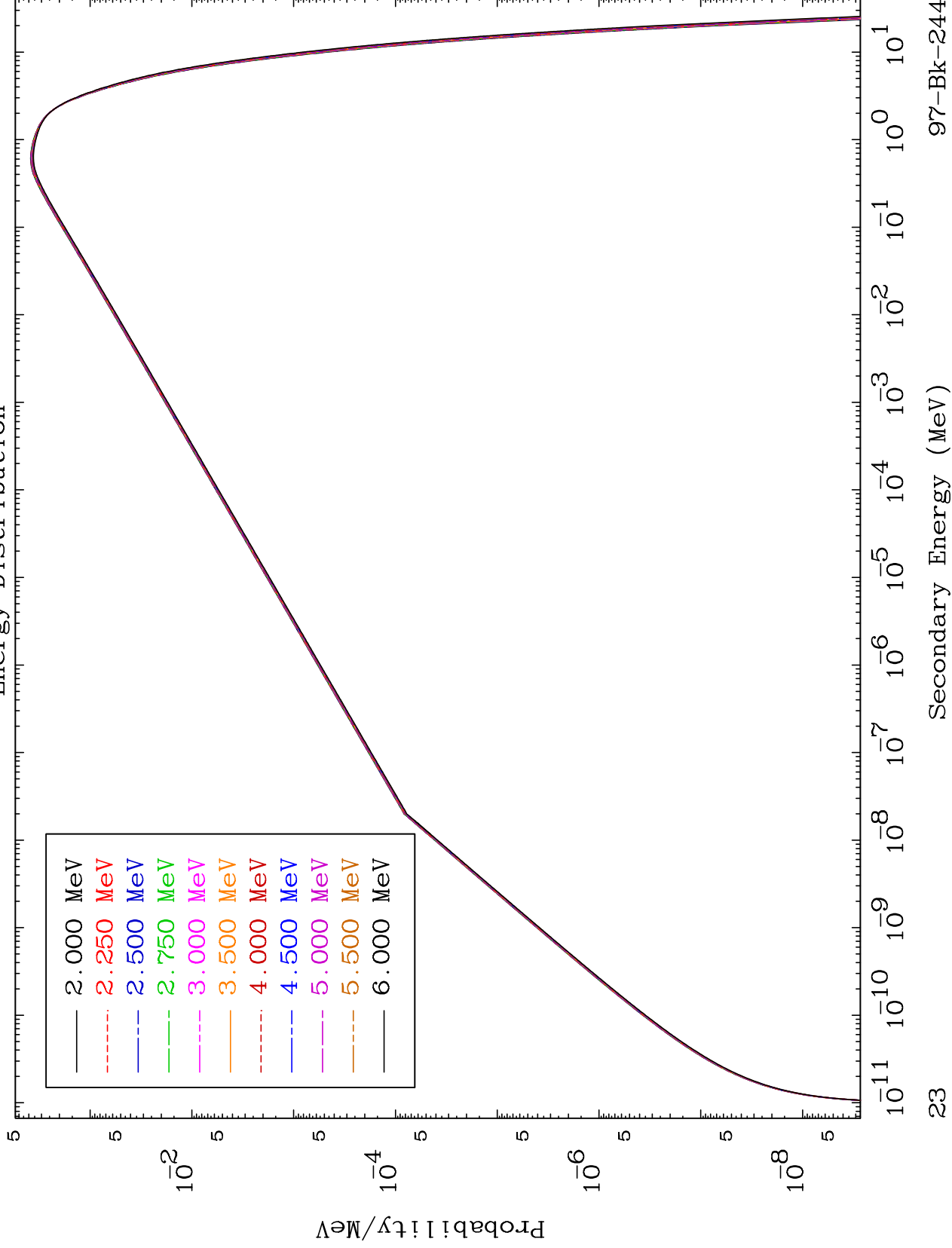
Secondary Energy (MeV)

22

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Fission Energy Distribution

97-Bk-244

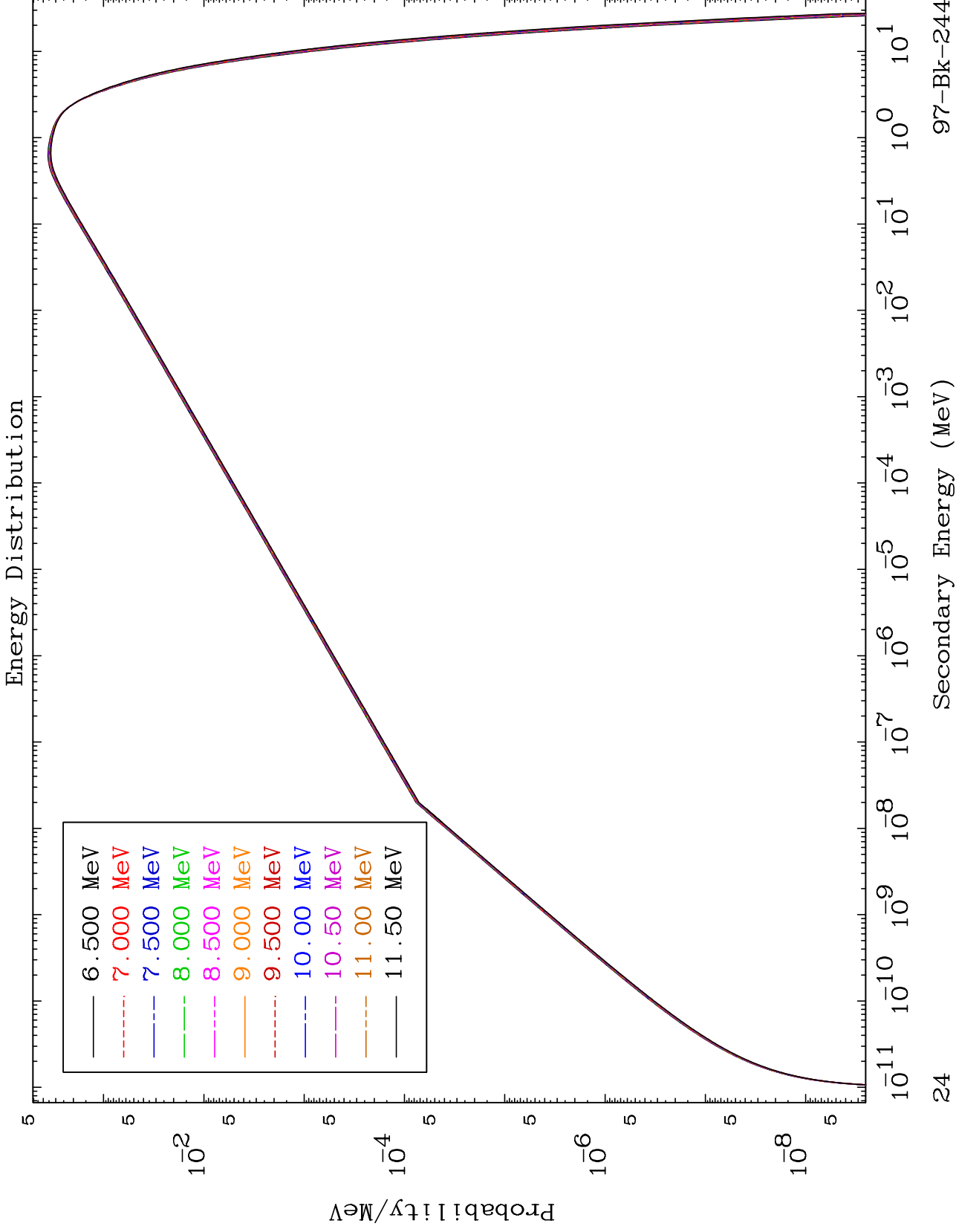




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Fission

97-Bk-244



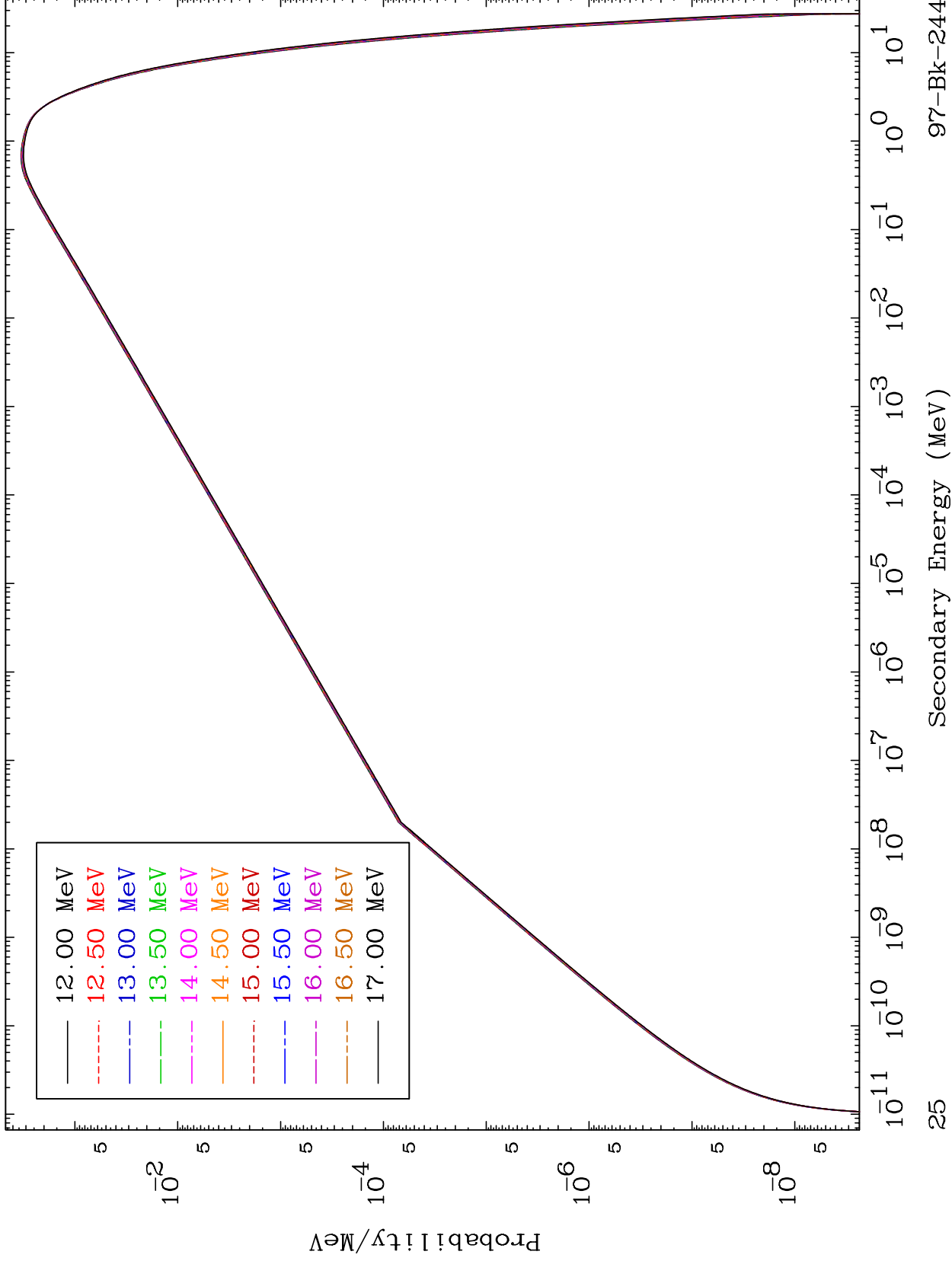
24

97-Bk-244

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### Fission Energy Distribution

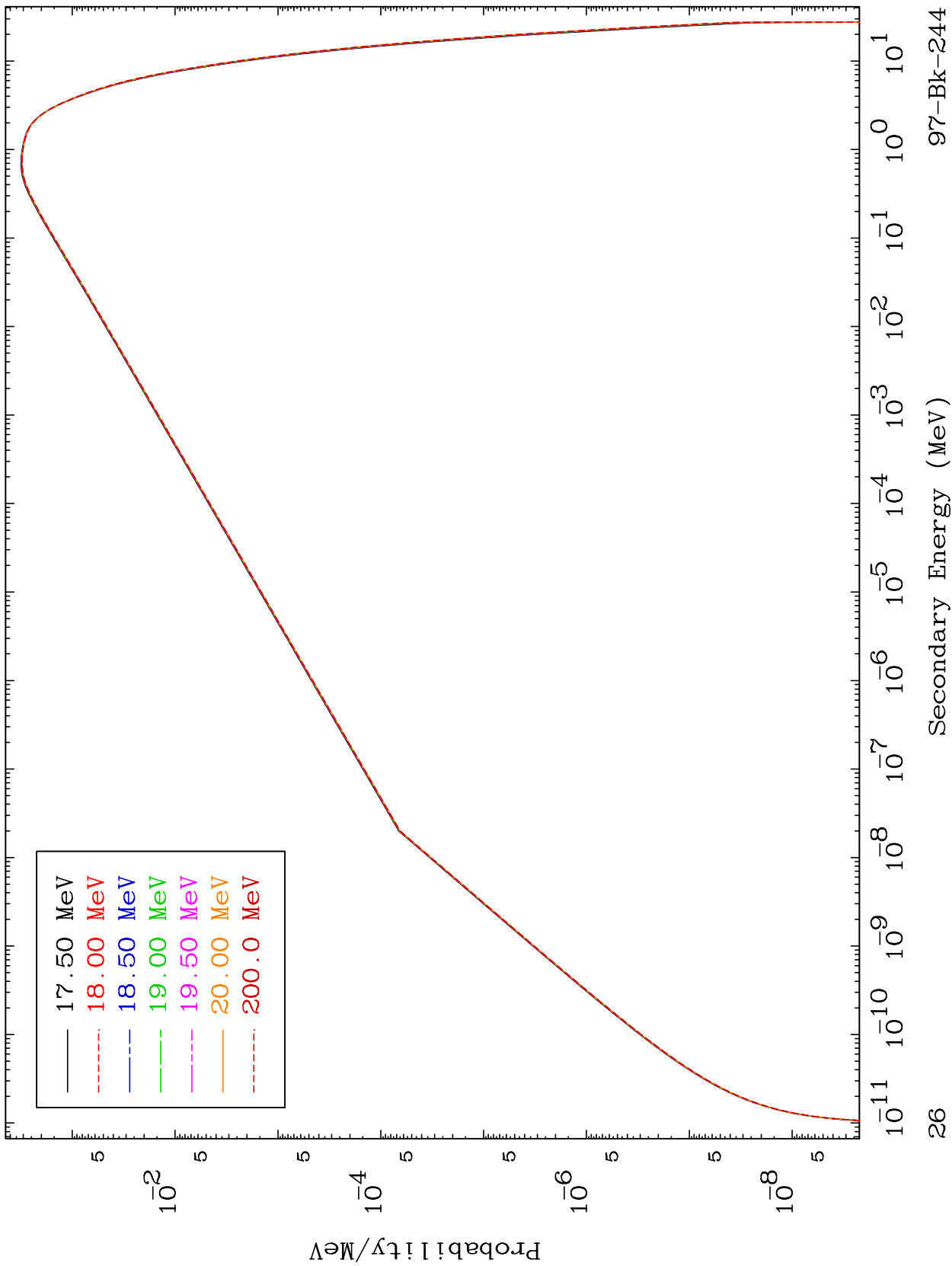
97-Bk-244



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### Fission Energy Distribution

97-Bk-244



97-Bk-244

Secondary Energy (MeV)

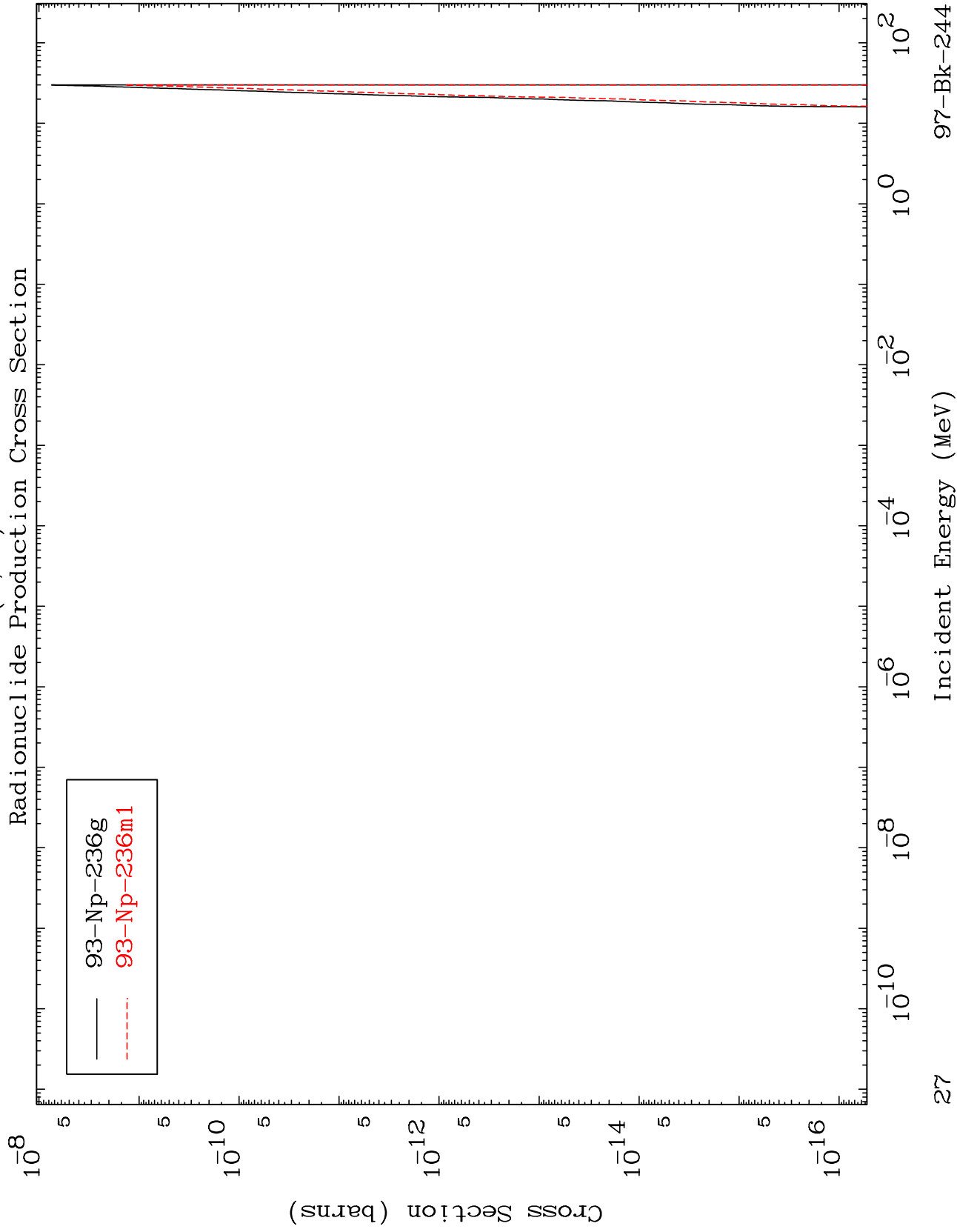
26

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(n,n') 2 $\alpha$

97-Bk-244

Radionuclide Production Cross Section



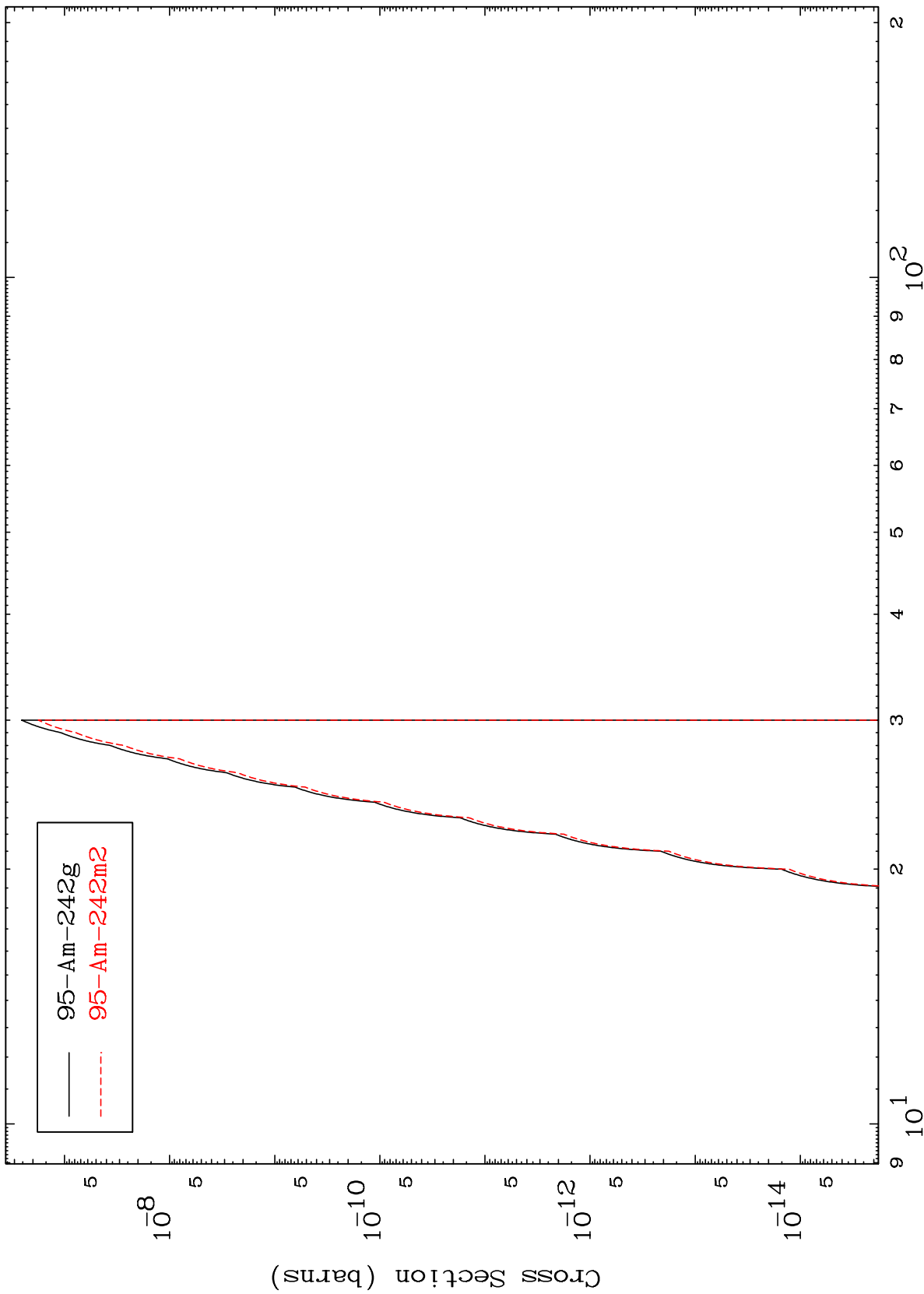
— 93-Np-236g  
- - - 93-Np-236m1

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(n,2n) p

97-Bk-244

Radionuclide Production Cross Section



28

Incident Energy (MeV)

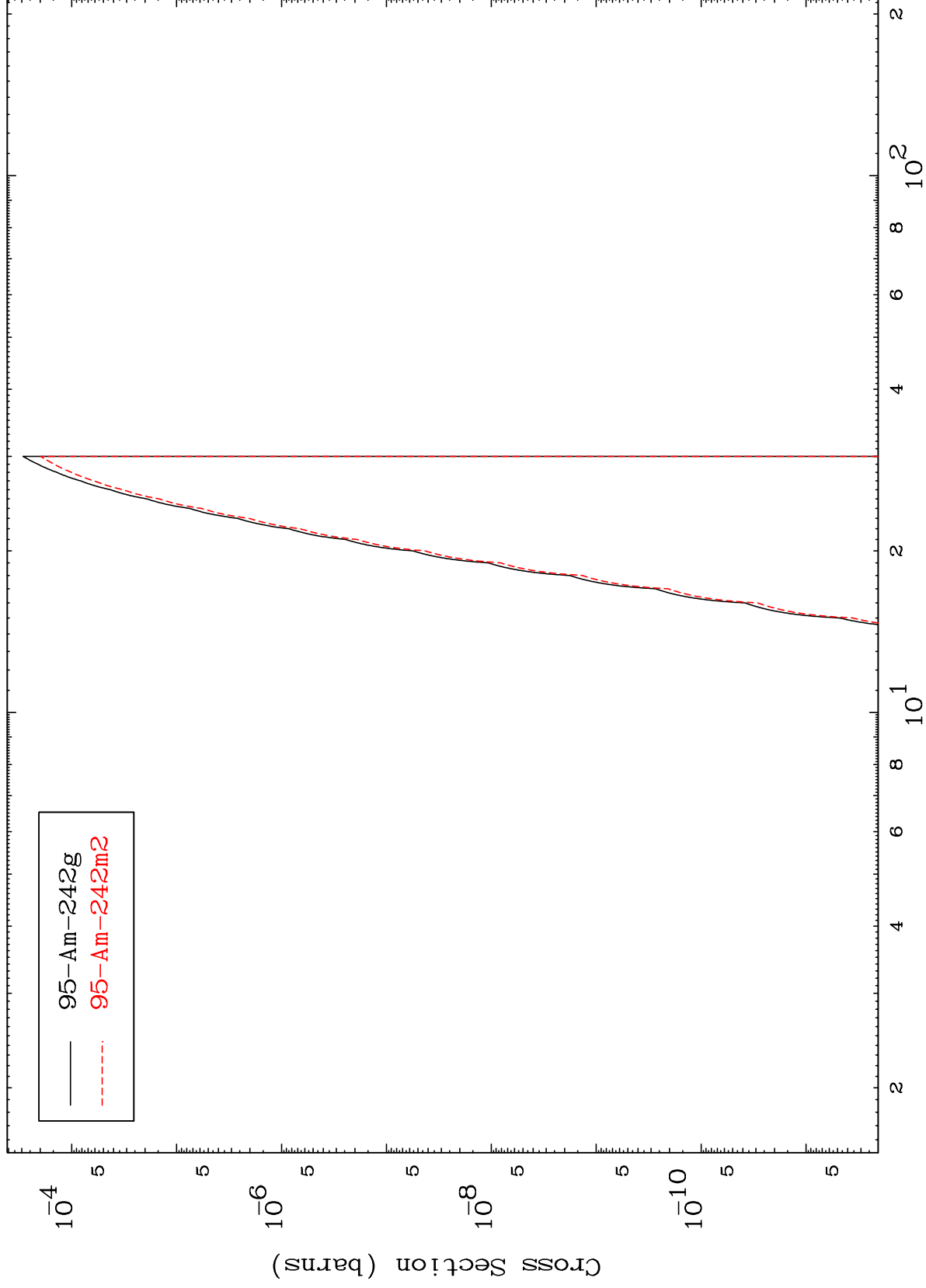
97-Bk-244

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(n,He-3)

97-Bk-244

Radionuclide Production Cross Section



29

Incident Energy (MeV)

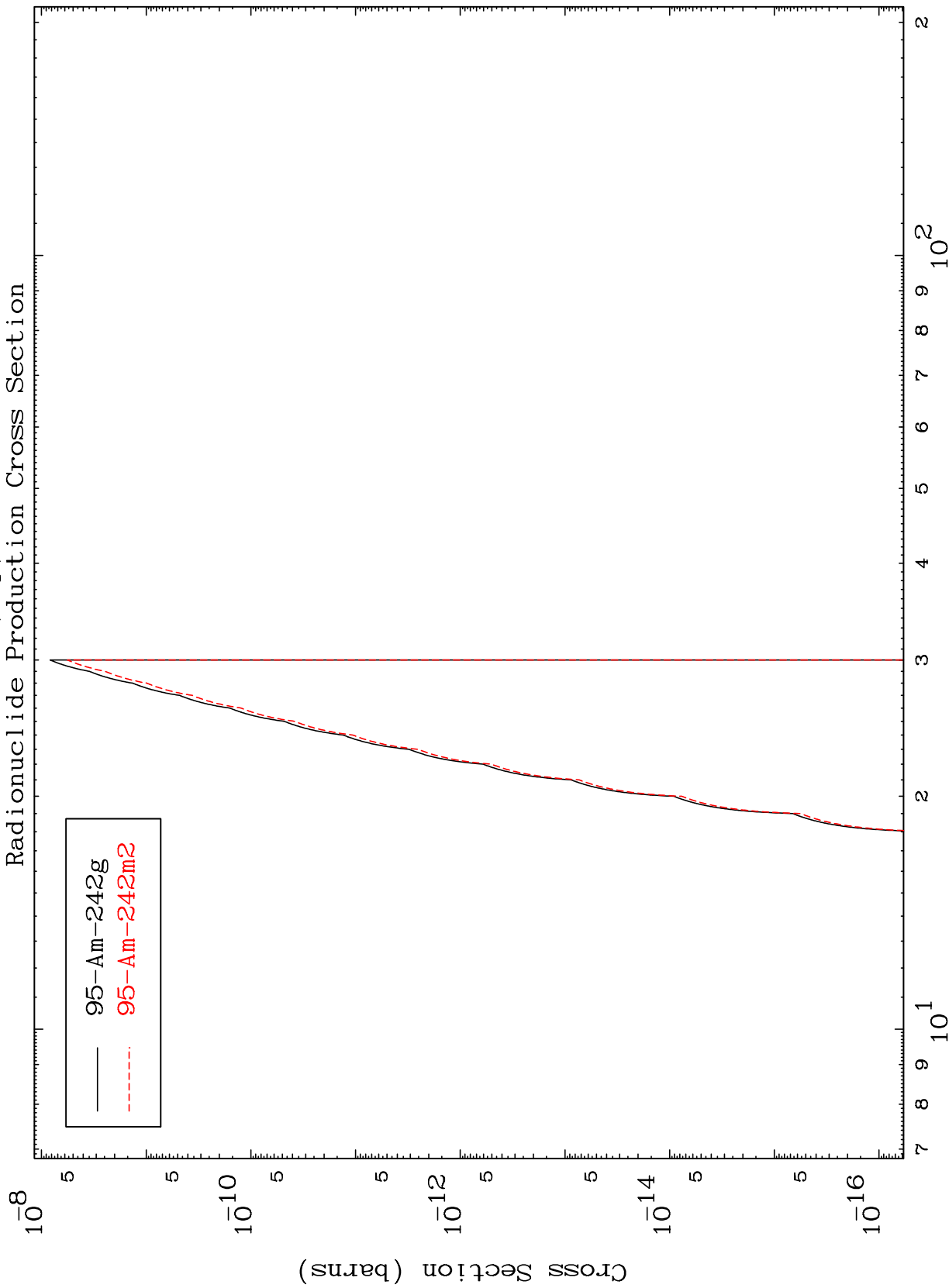
97-Bk-244

MAT 9737

(n,p) d

97-Bk-244

Radionuclide Production Cross Section



30

Incident Energy (MeV)

97-Bk-244