

Program EVALPLOT  
(Version 2018-1)

by

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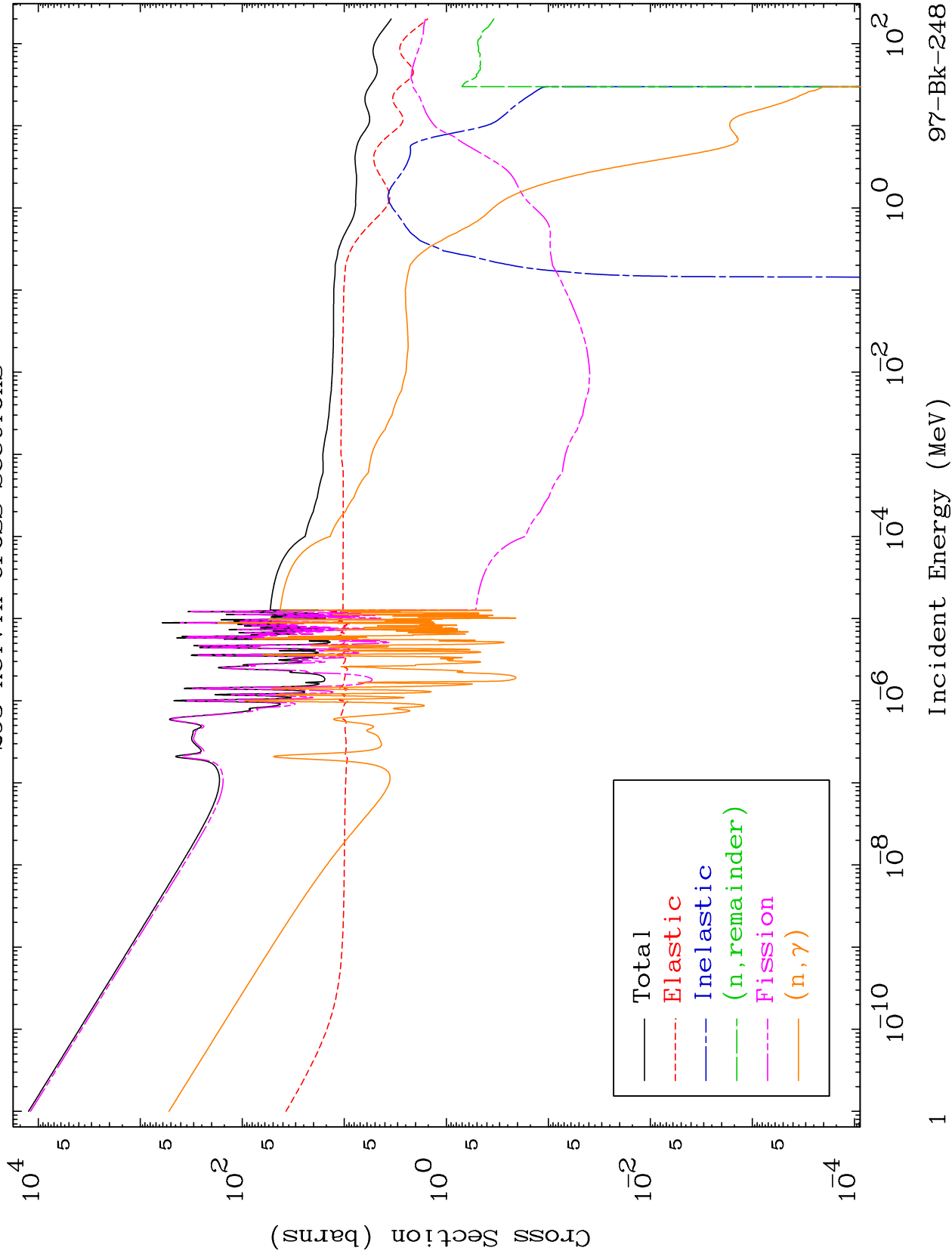
Press Mouse Button to Start

MAT 9749

Major

293 Kelvin Cross Sections

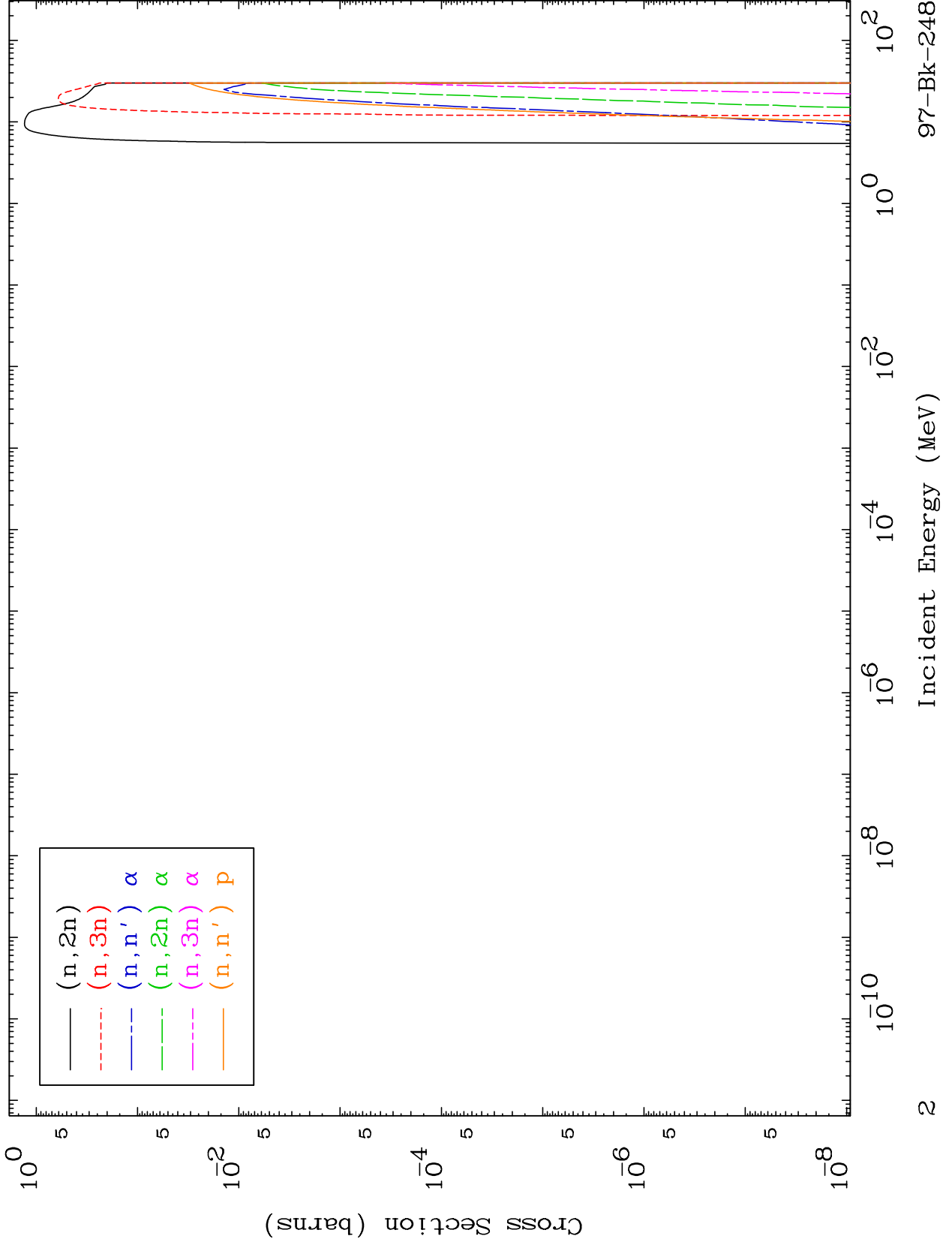
97-Bk-248

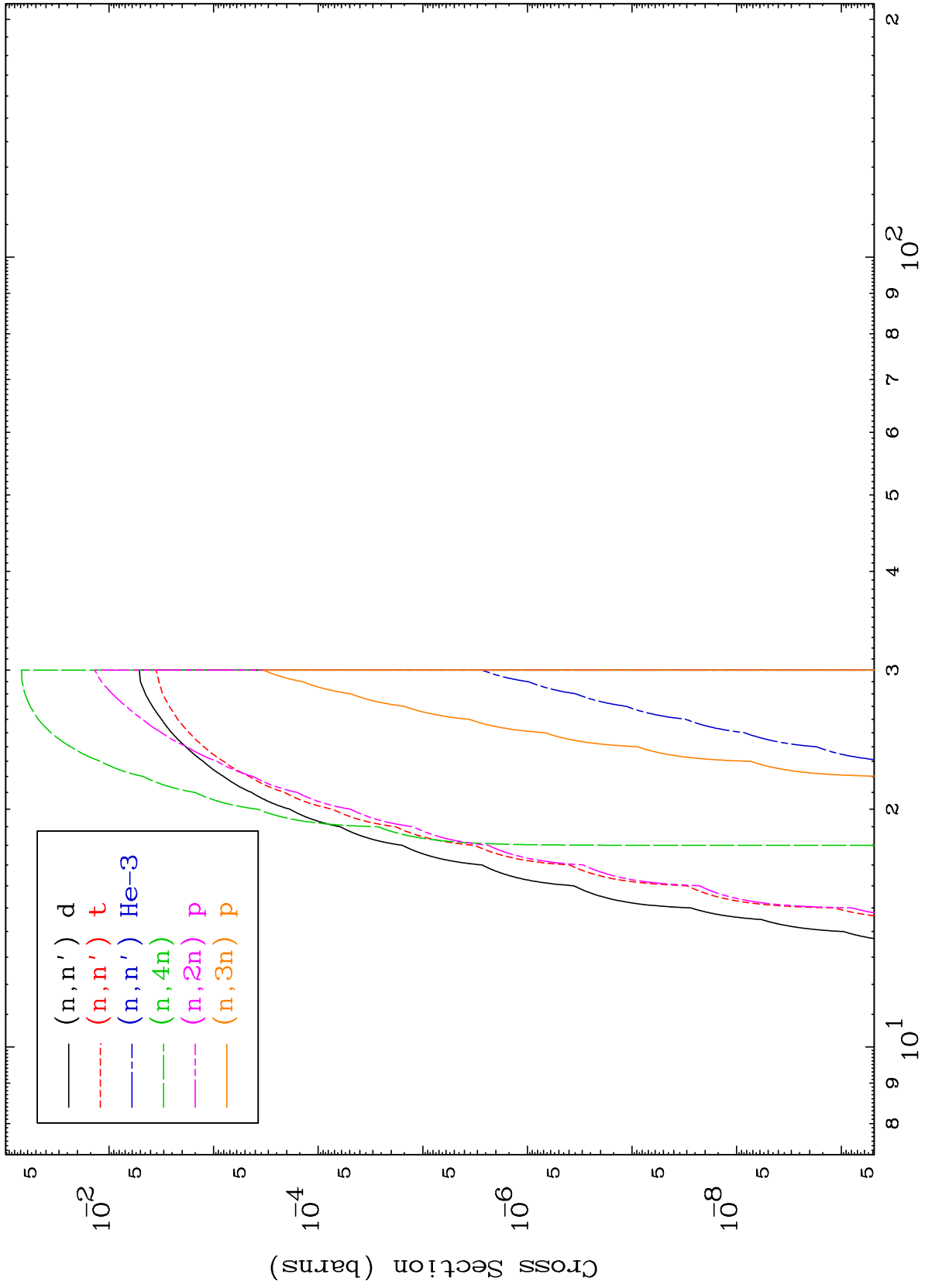


MAT 9749

Neutron Production  
293 Kelvin Cross Sections

97-Bk-248

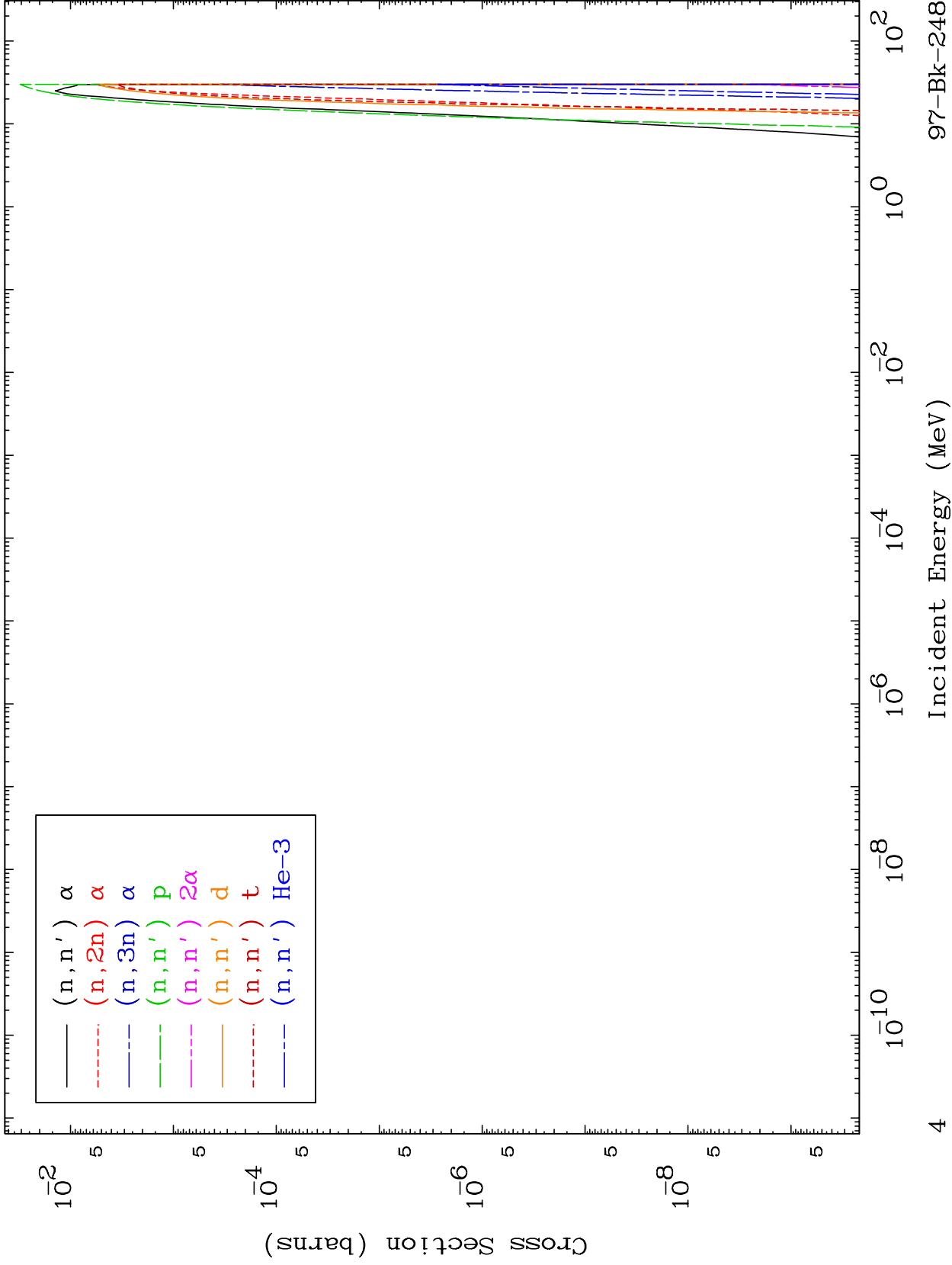




MAT 9749

Charged Particle  
293 Kelvin Cross Sections

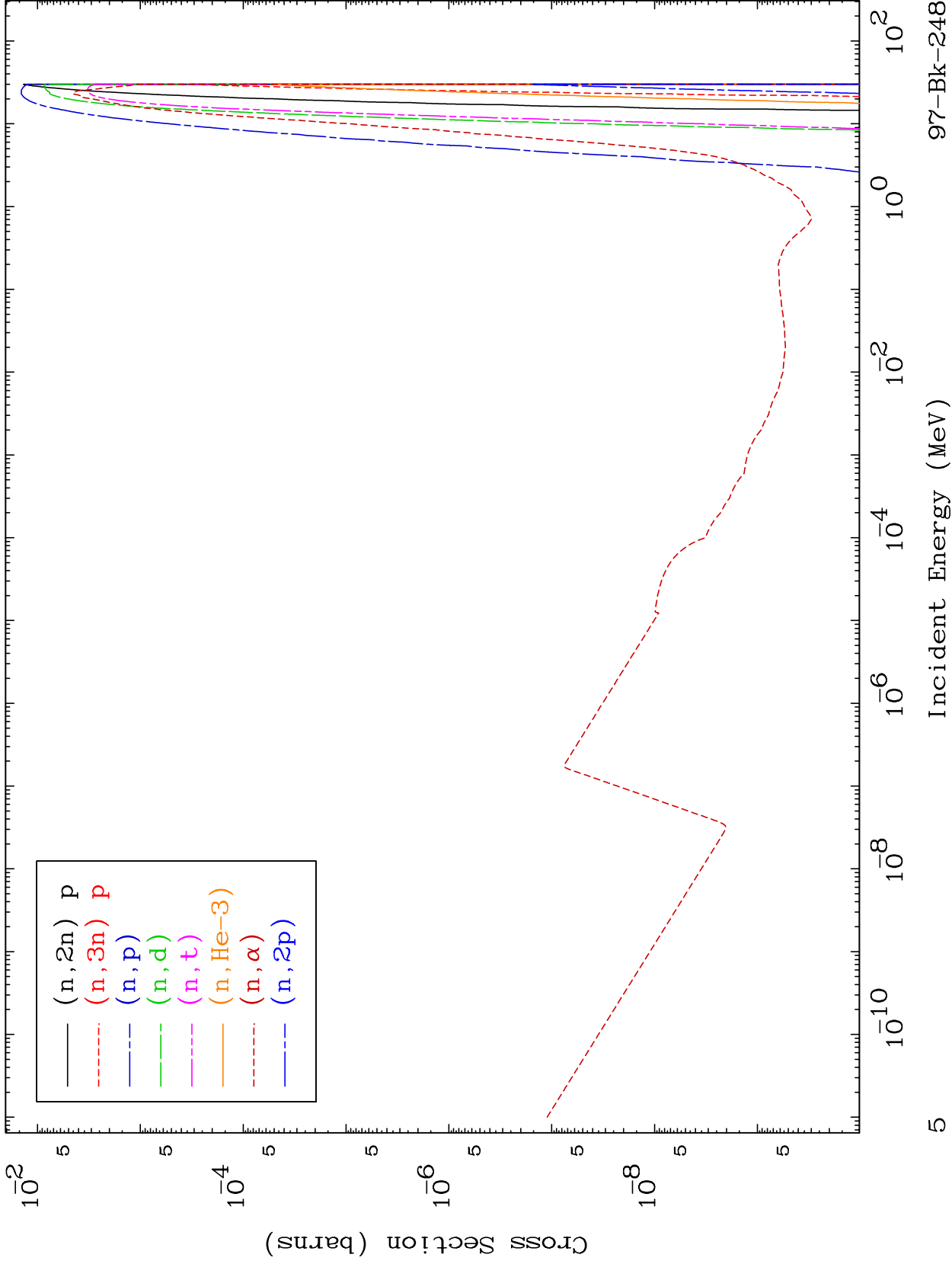
97-Bk-248



MAT 9749

Charged Particle  
293 Kelvin Cross Sections

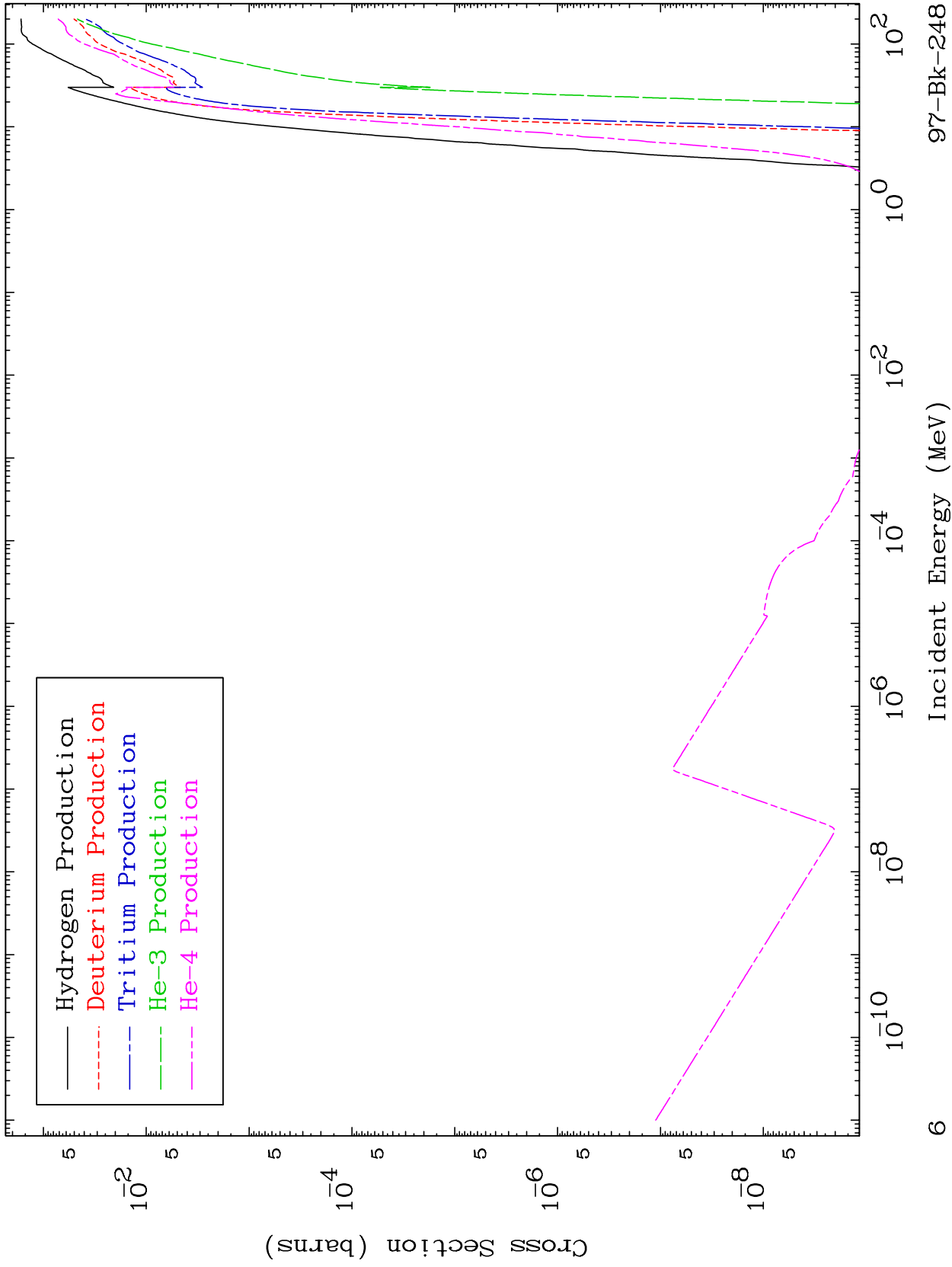
97-Bk-248



MAT 9749

Particle Production  
293 Kelvin Cross Sections

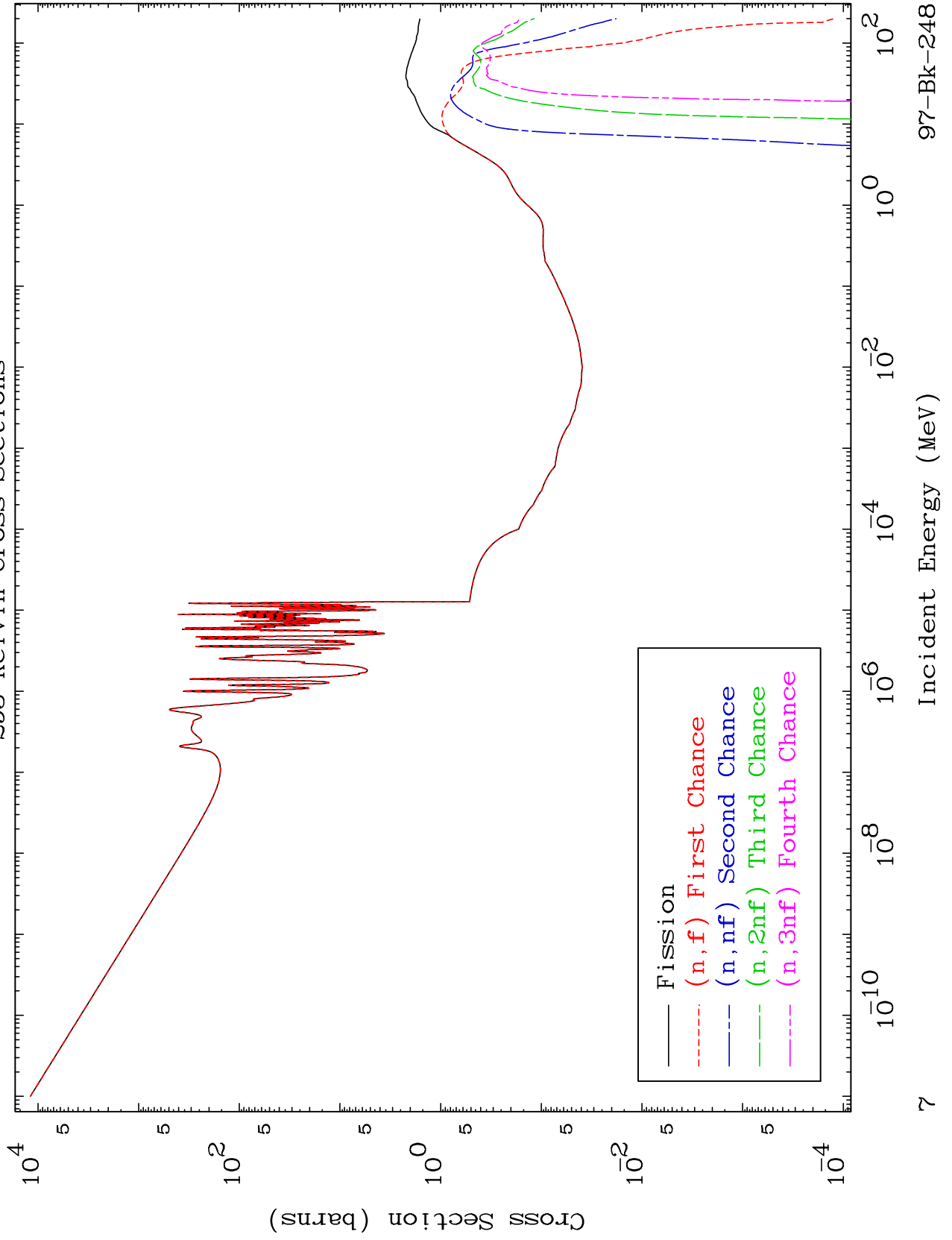
97-Bk-248



MAT 9749

Fission  
293 Kelvin Cross Sections

97-Bk-248





MAT 9749

(n,n') Level

97-Bk-248

293 Kelvin Cross Sections

$10^6$

$10^4$

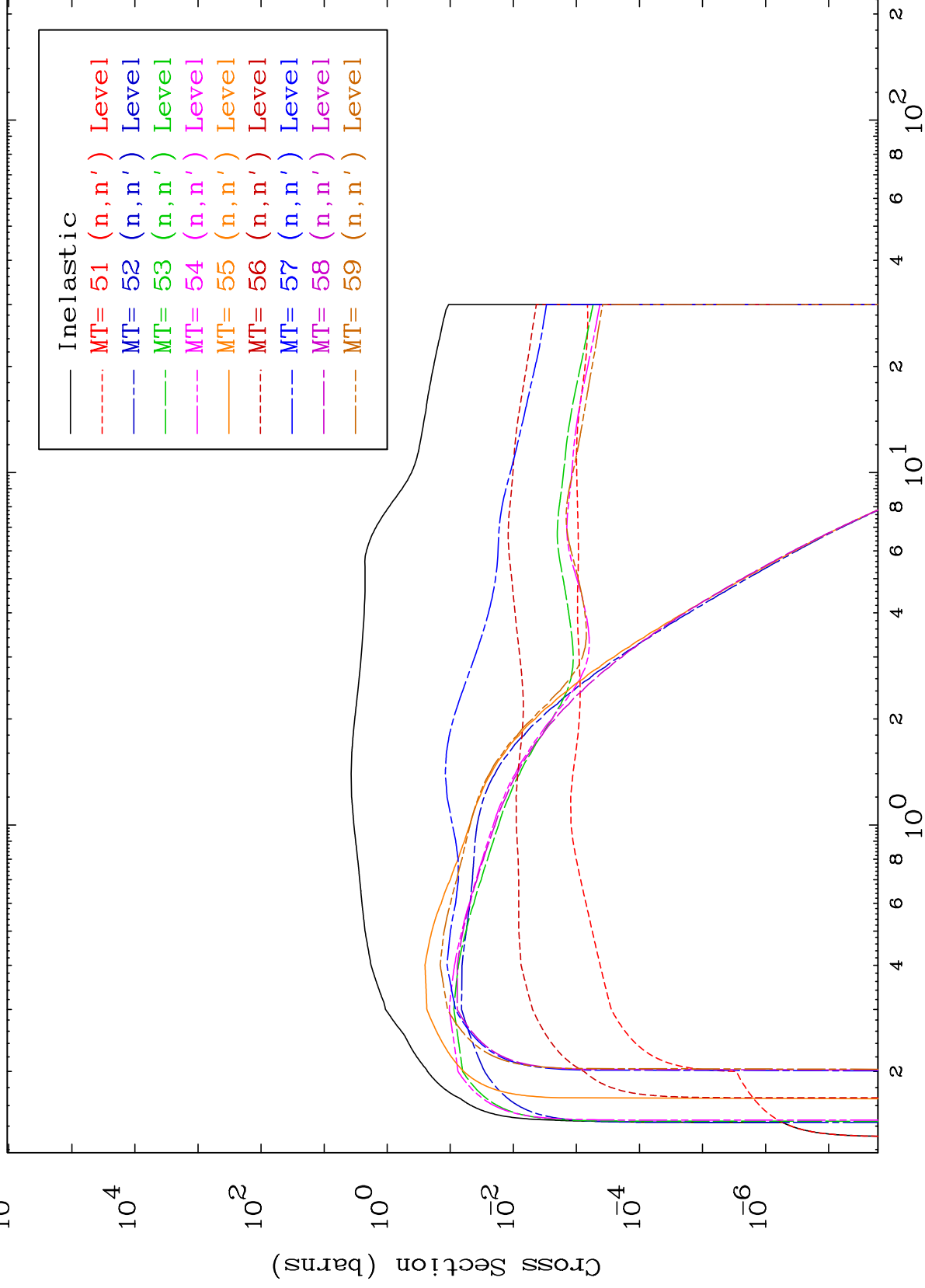
$10^2$

$10^0$

$10^{-2}$

$10^{-4}$

$10^{-6}$

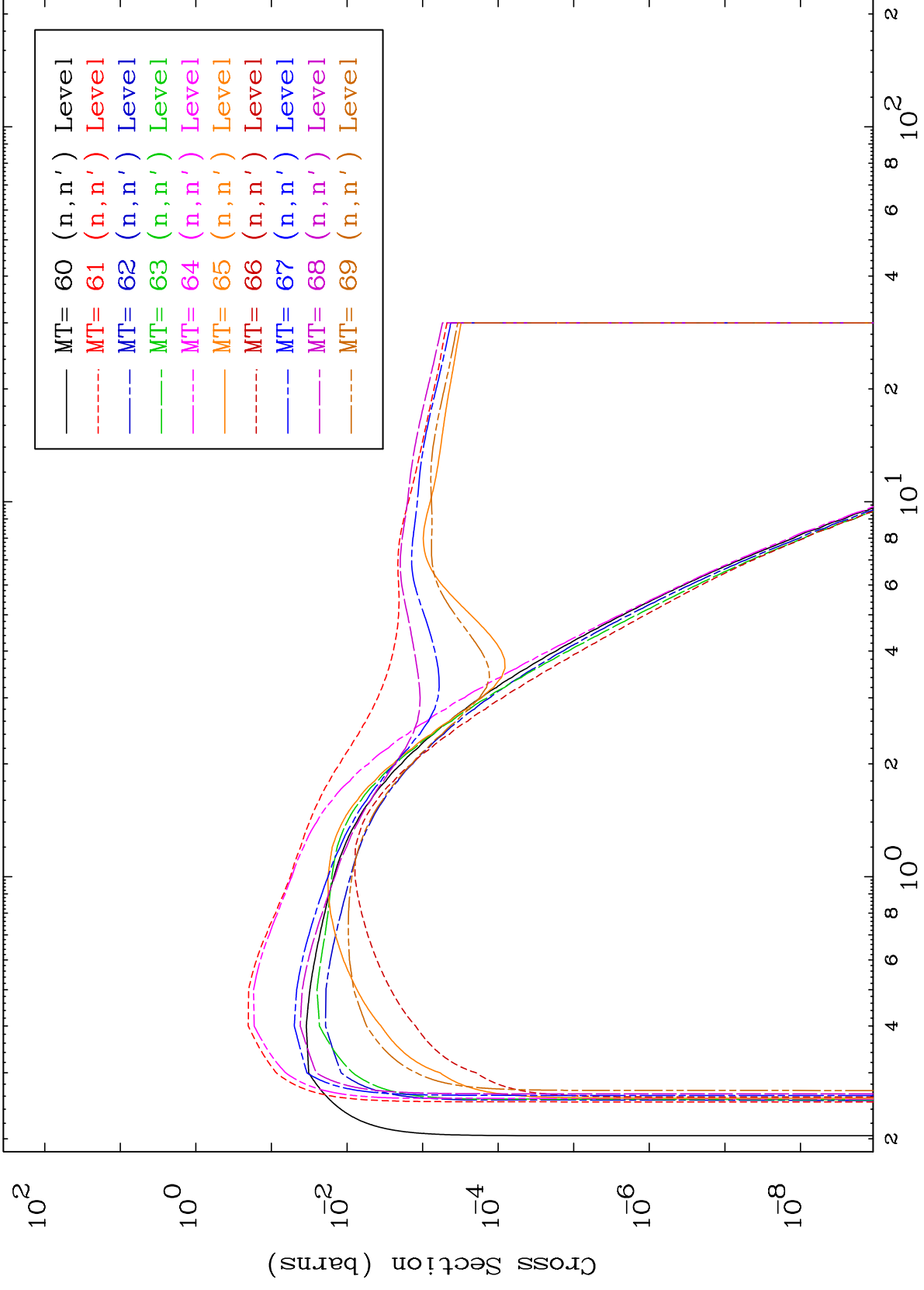


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(n,n') Level

97-Bk-248

293 Kelvin Cross Sections

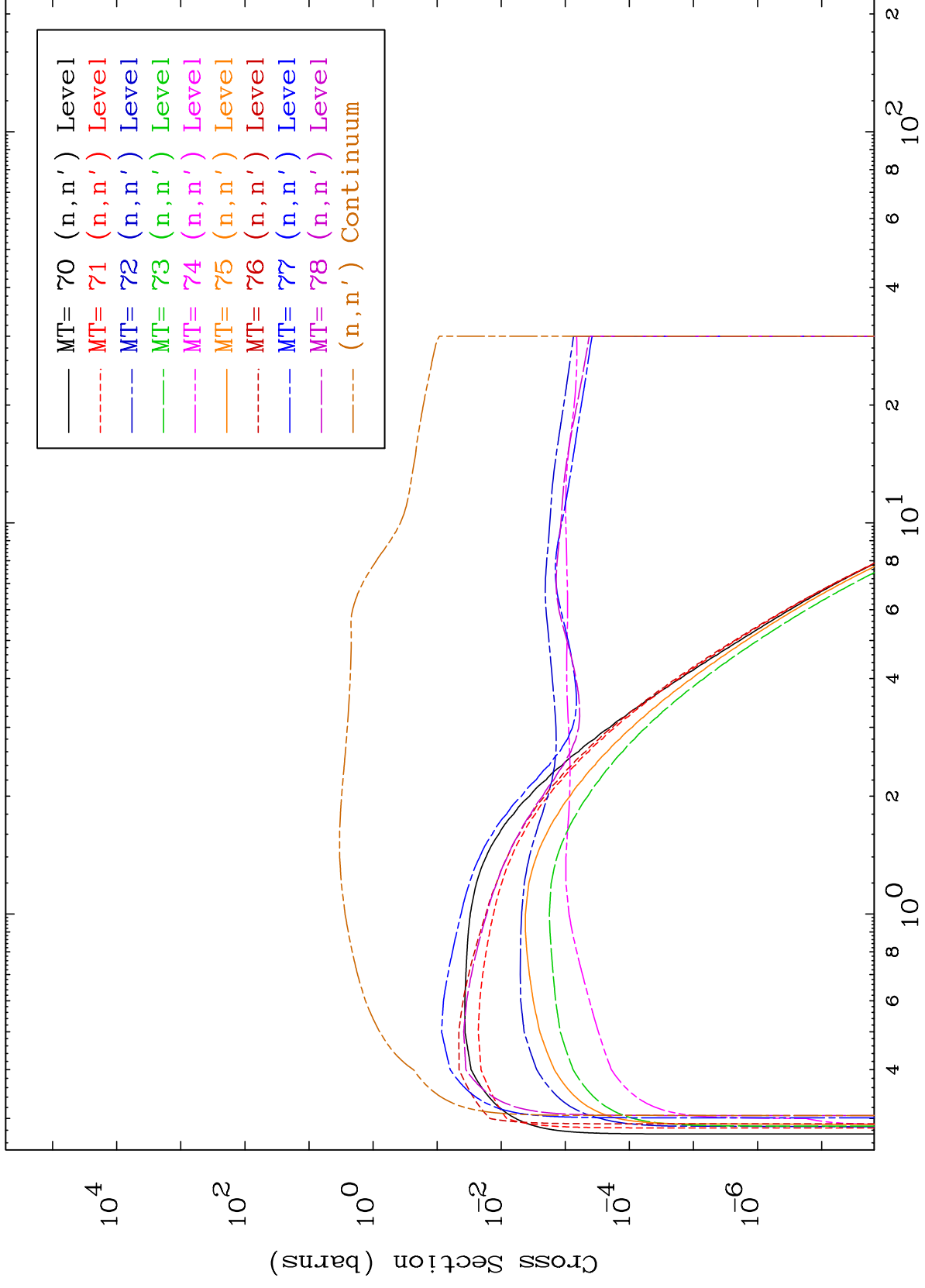


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(n,n') Level

97-Bk-248

293 Kelvin Cross Sections



10

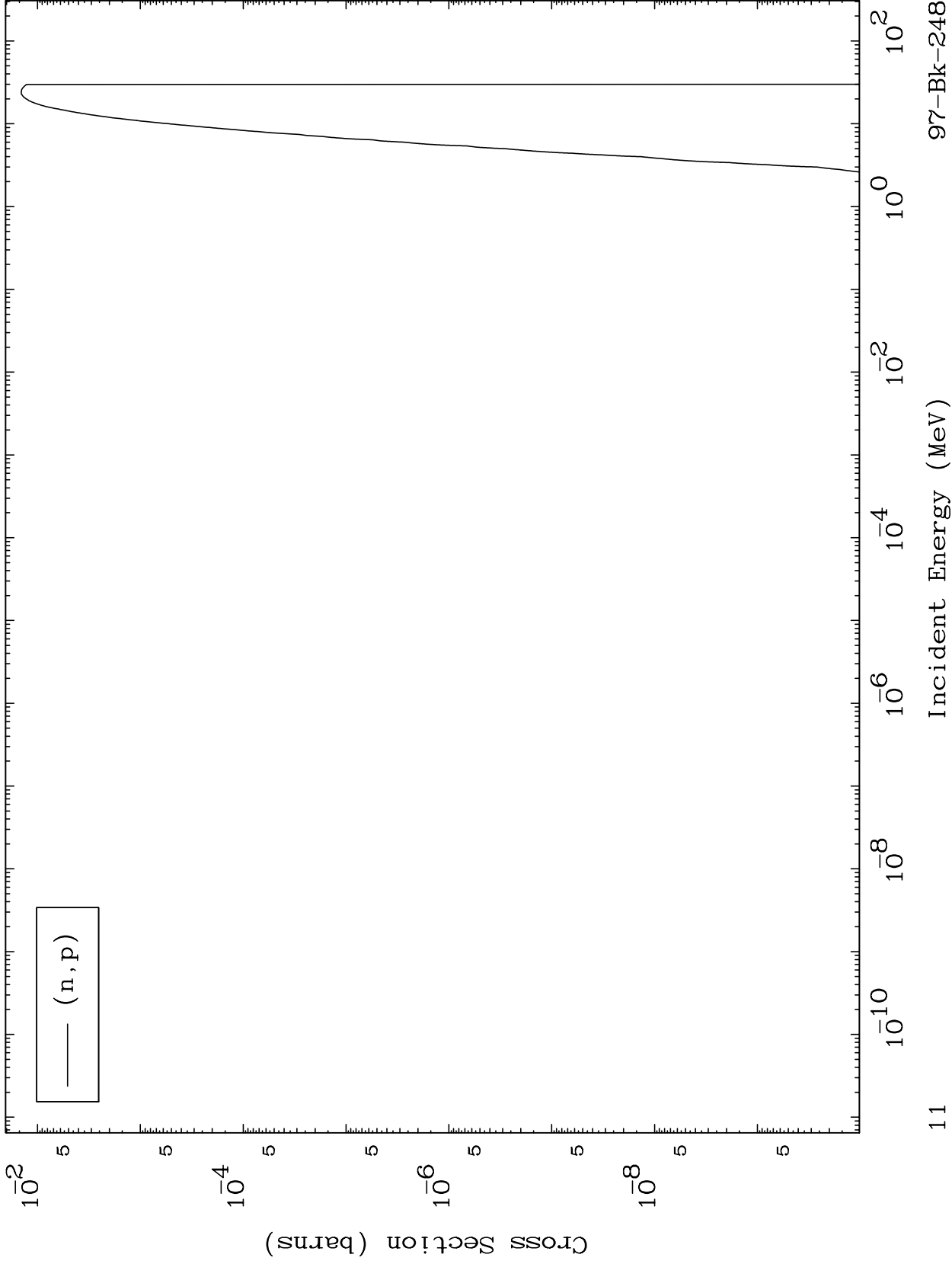
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,p) Levels  
293 Kelvin Cross Sections

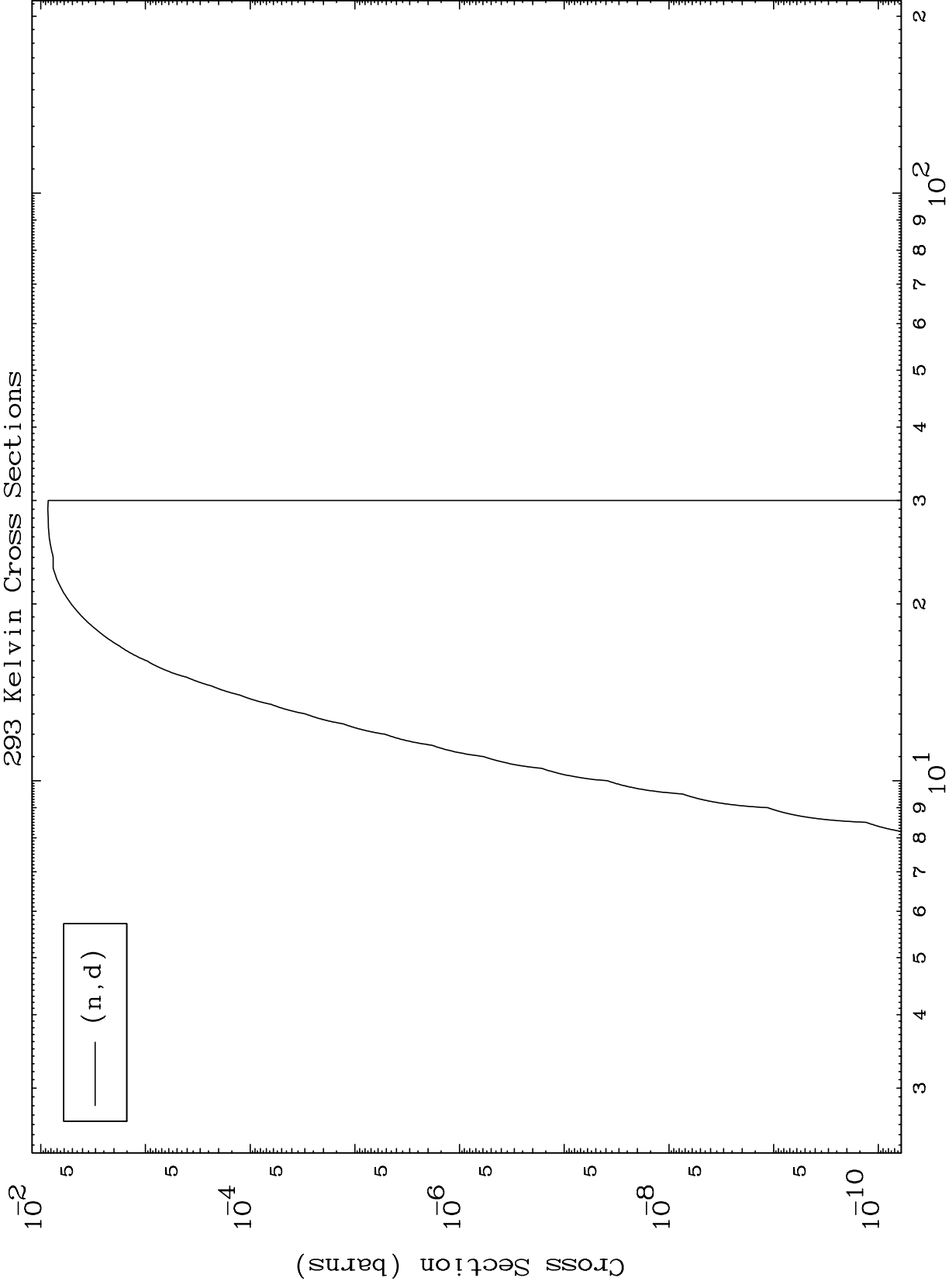
97-Bk-248



MAT 9749

(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-248



12

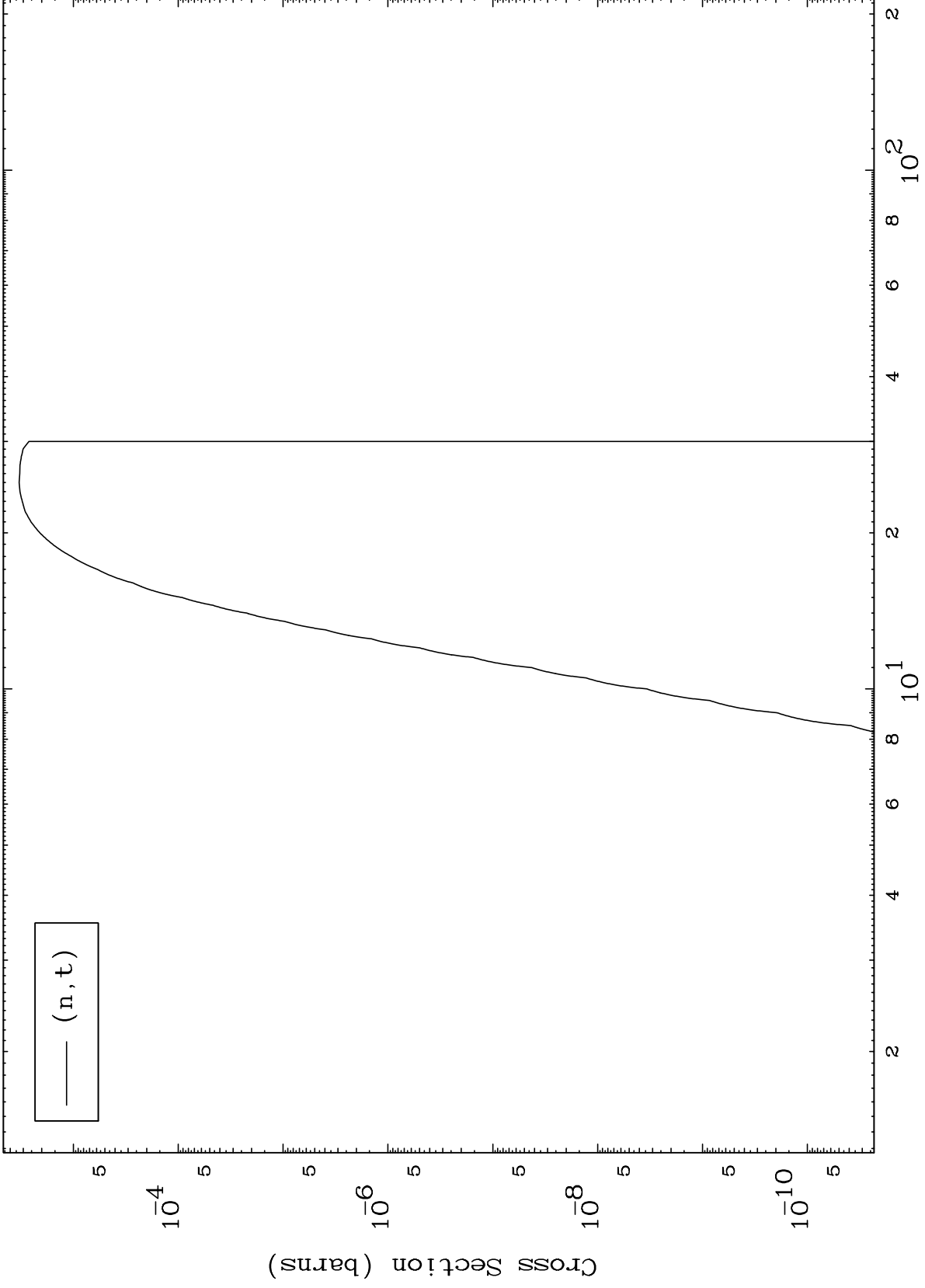
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-248



13

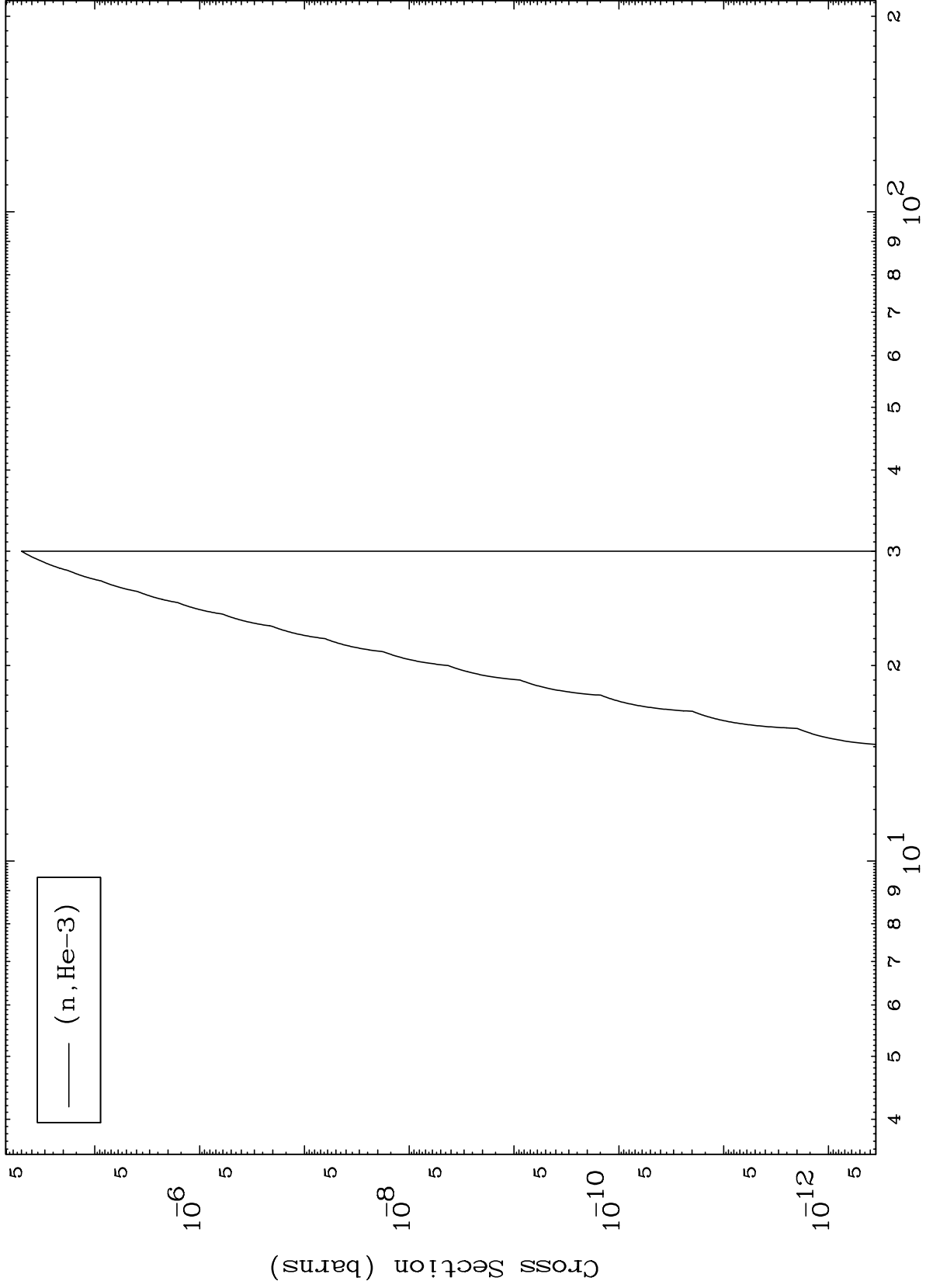
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-248



14

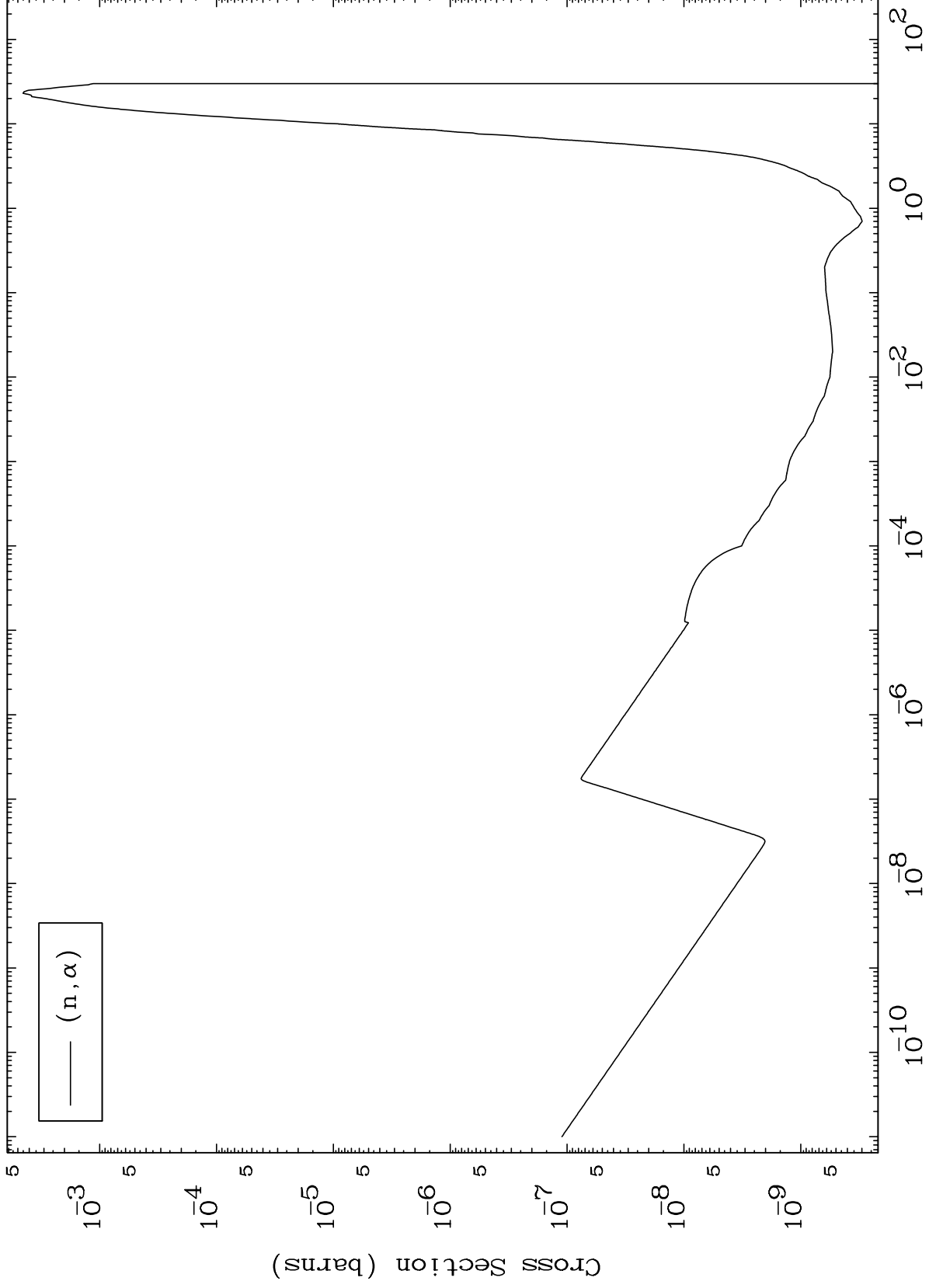
Incident Energy (MeV)

97-Bk-248

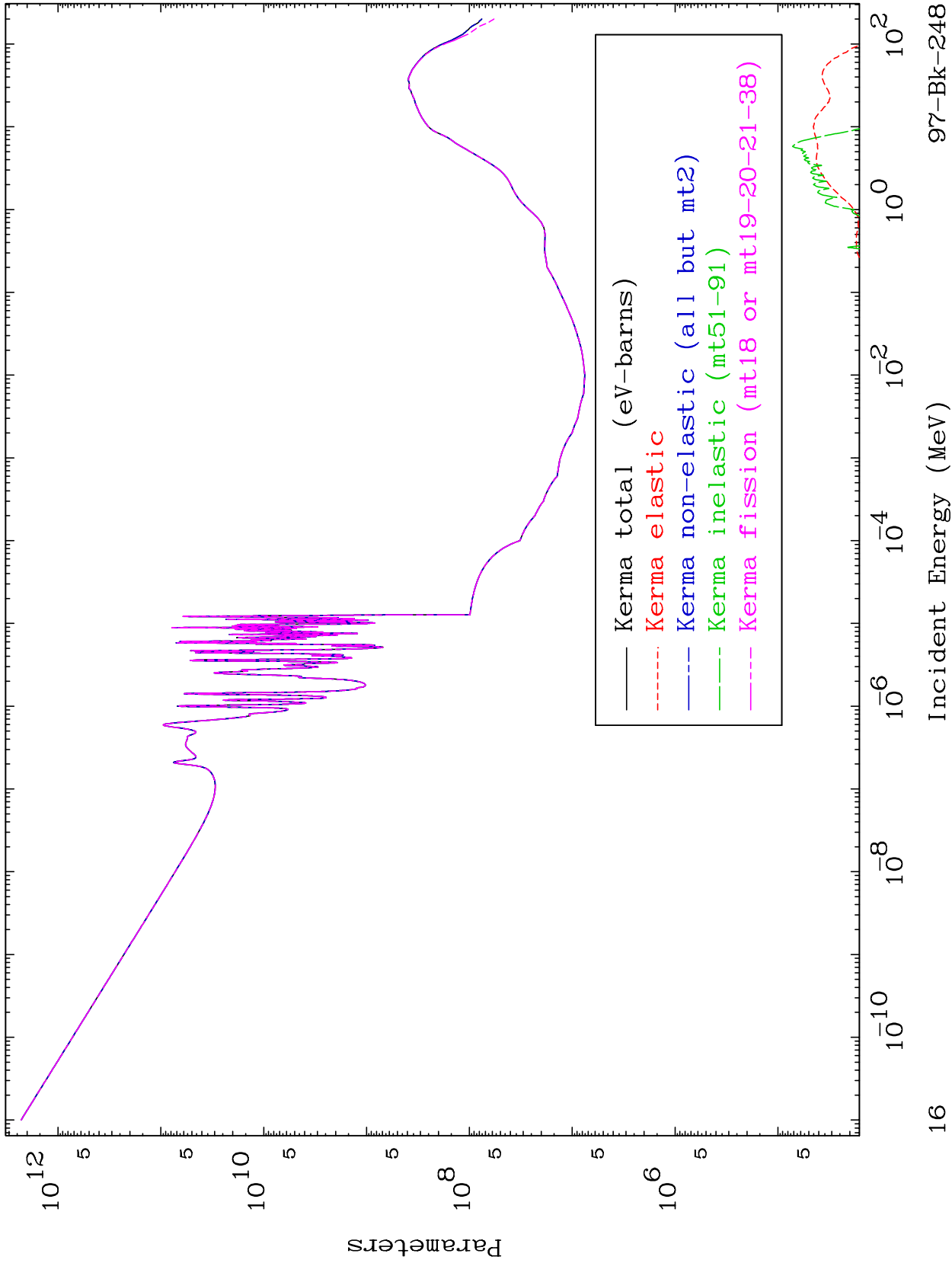
MAT 9749

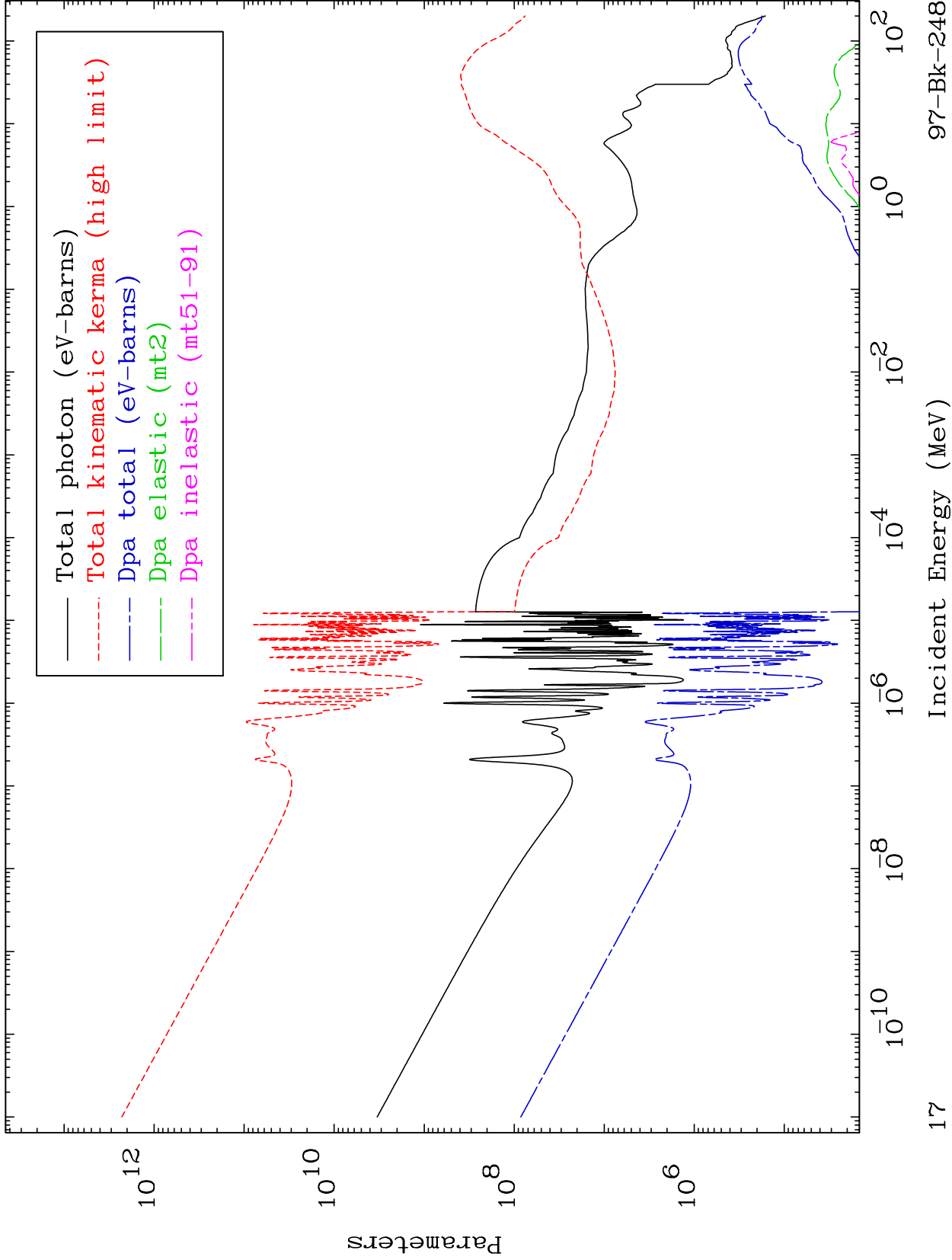
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

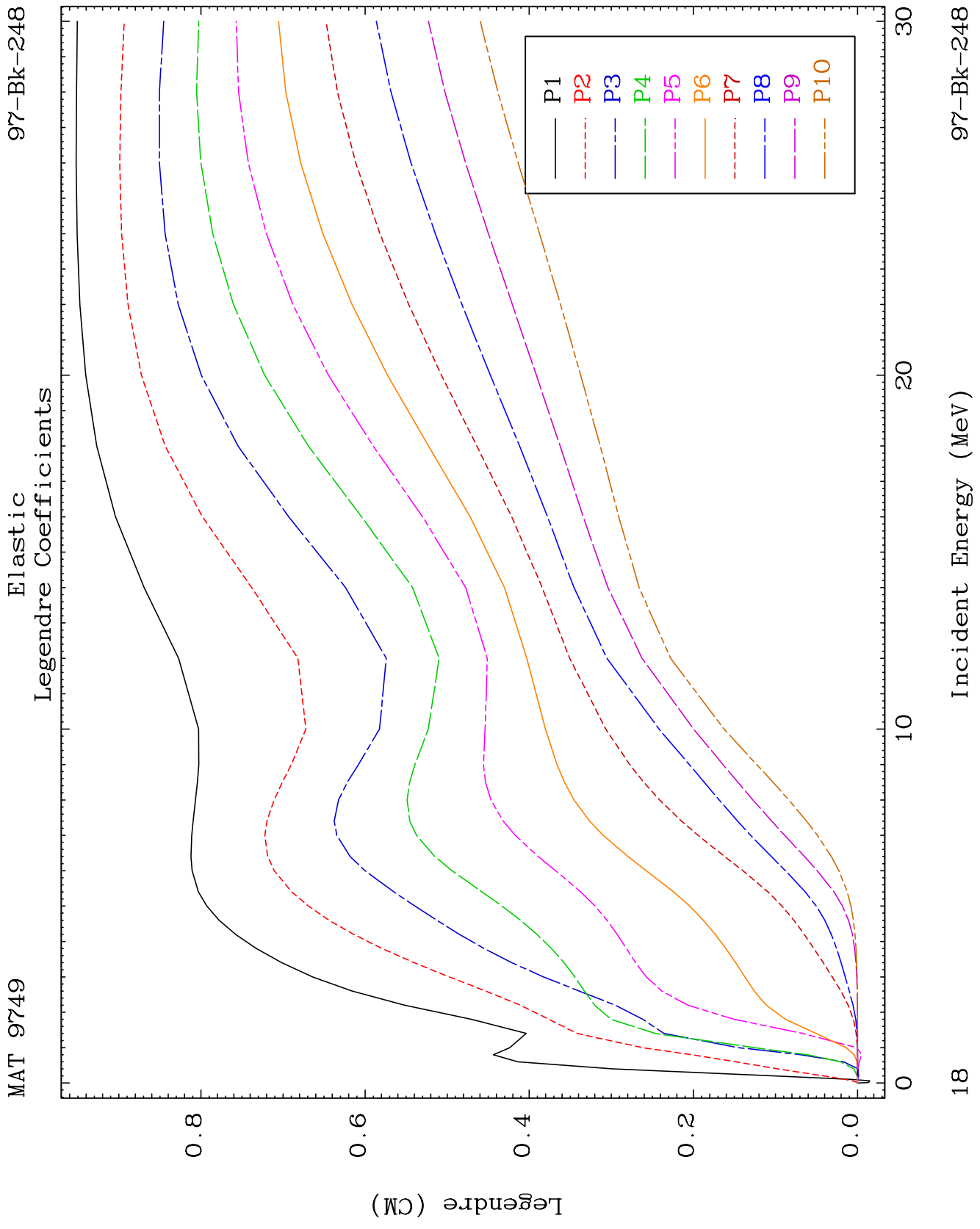
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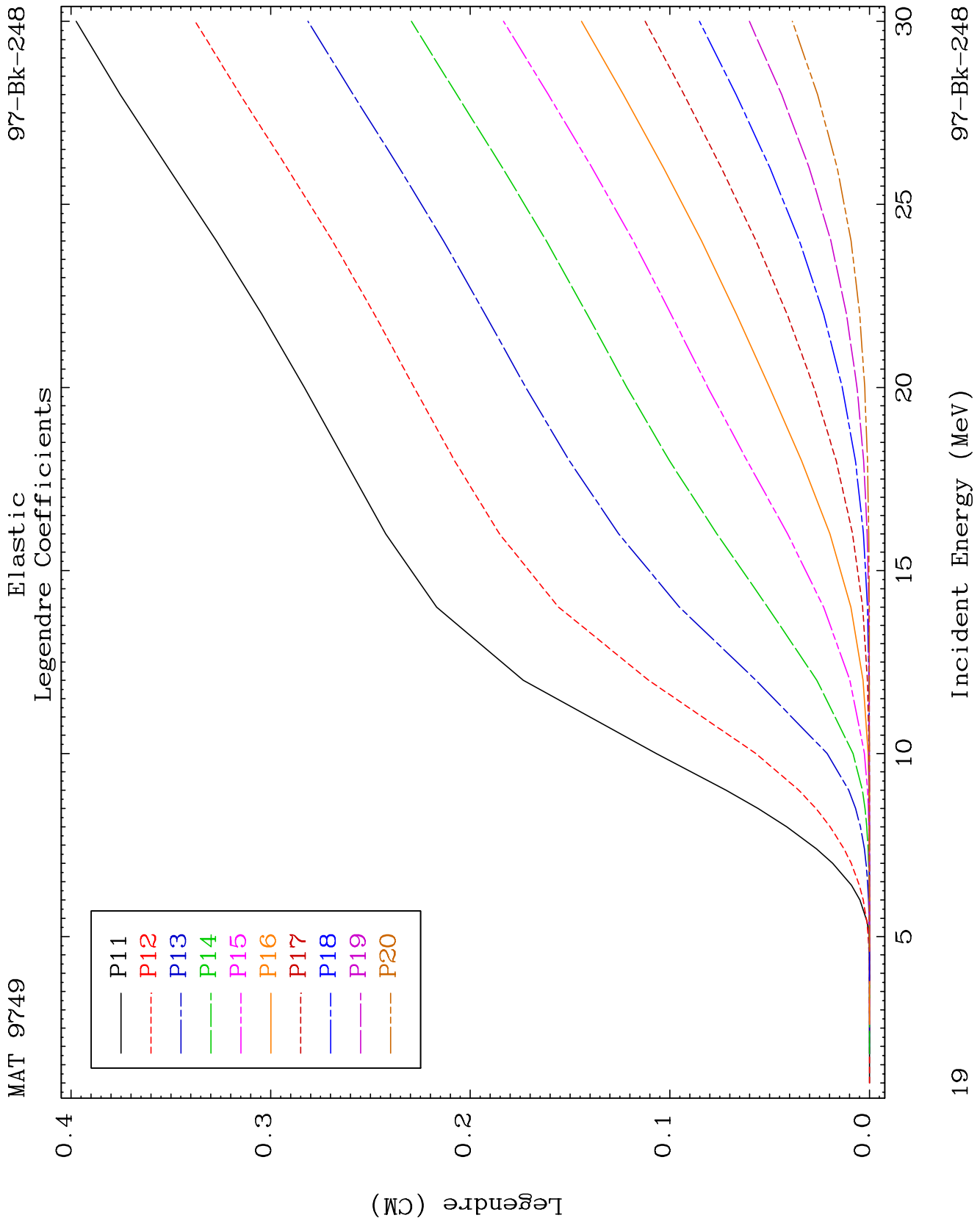








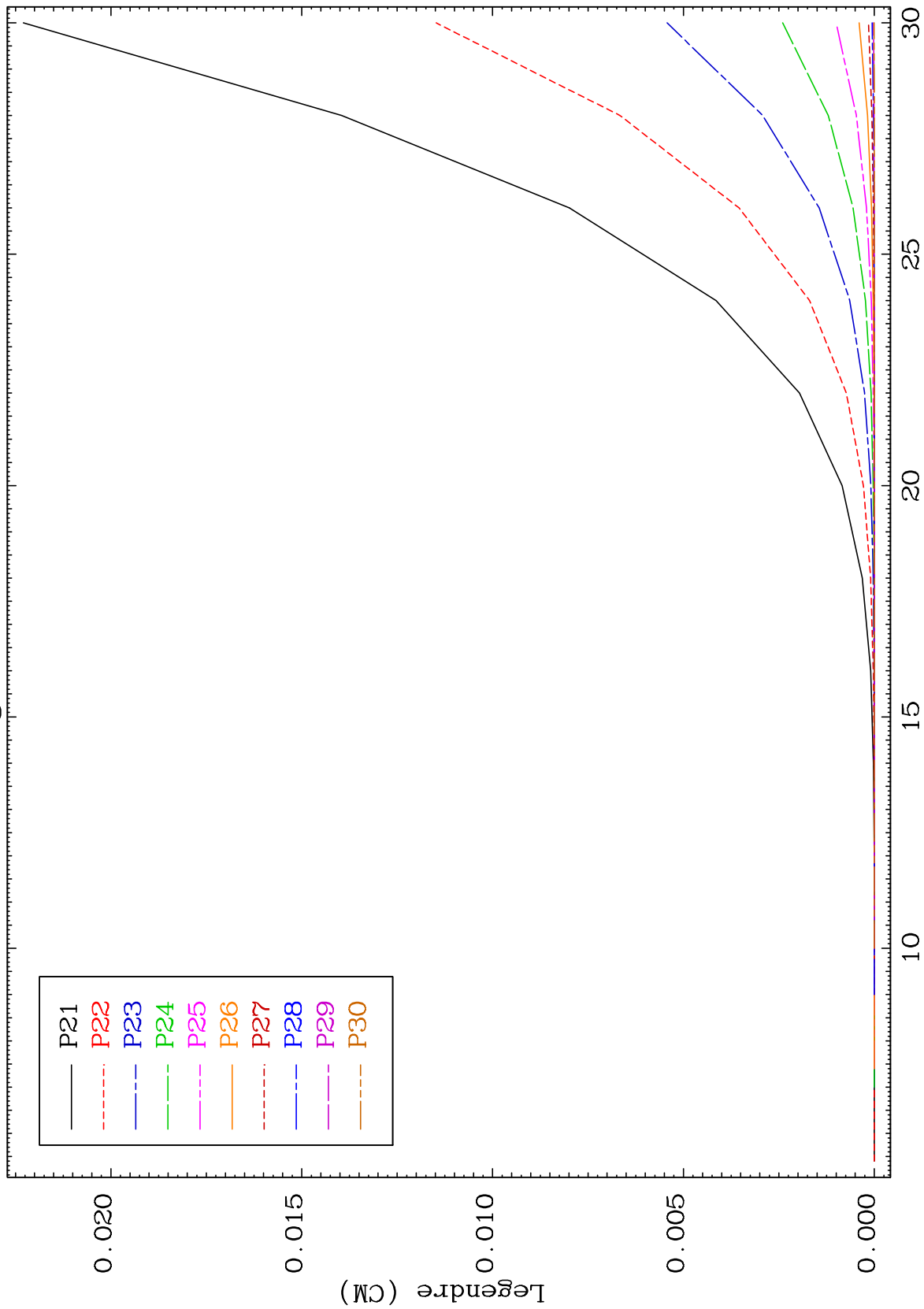




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### Elastic Legendre Coefficients

97-Bk-248



20

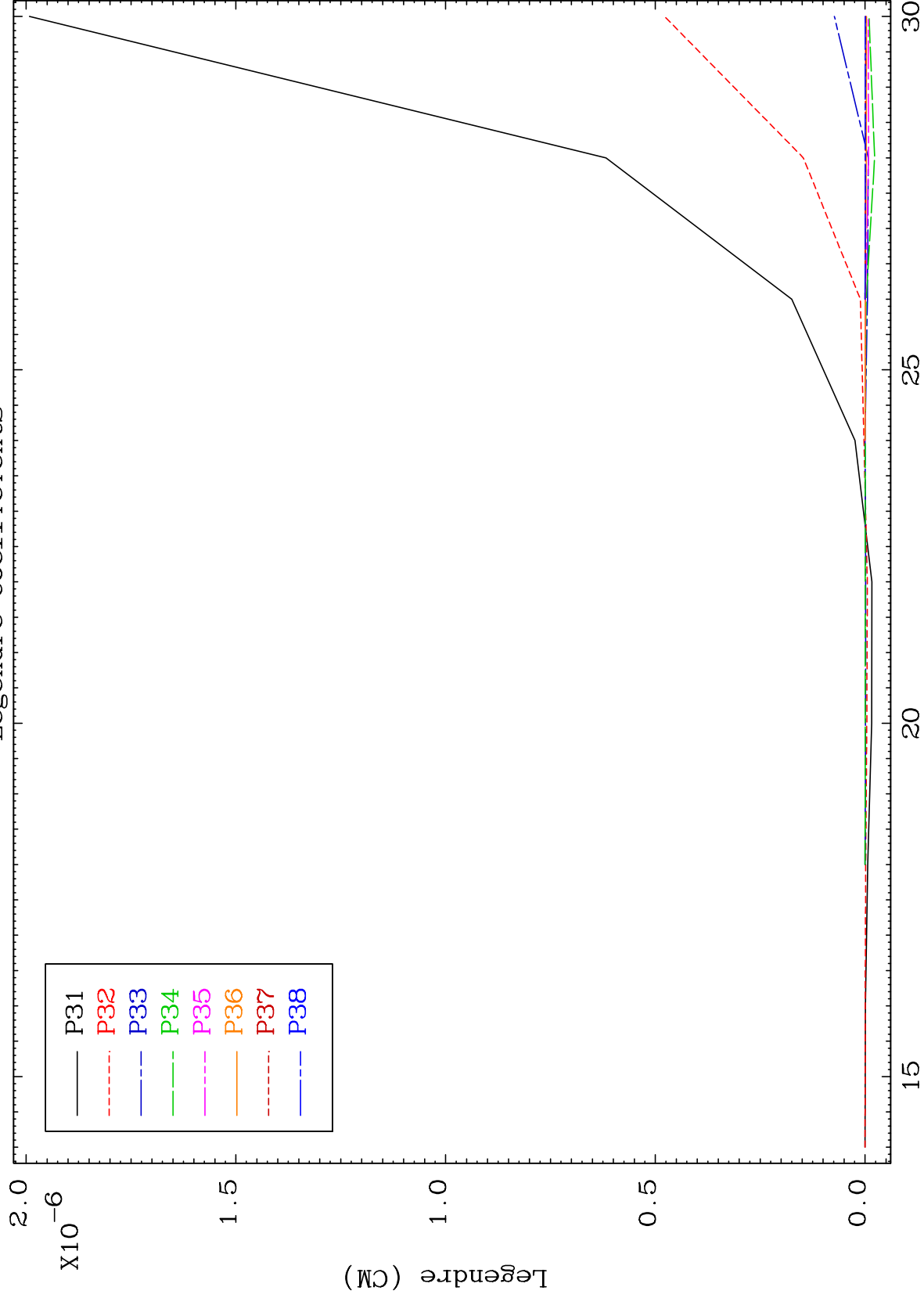
Incident Energy (MeV)

97-Bk-248

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Elastic  
Legendre Coefficients

97-Bk-248



21

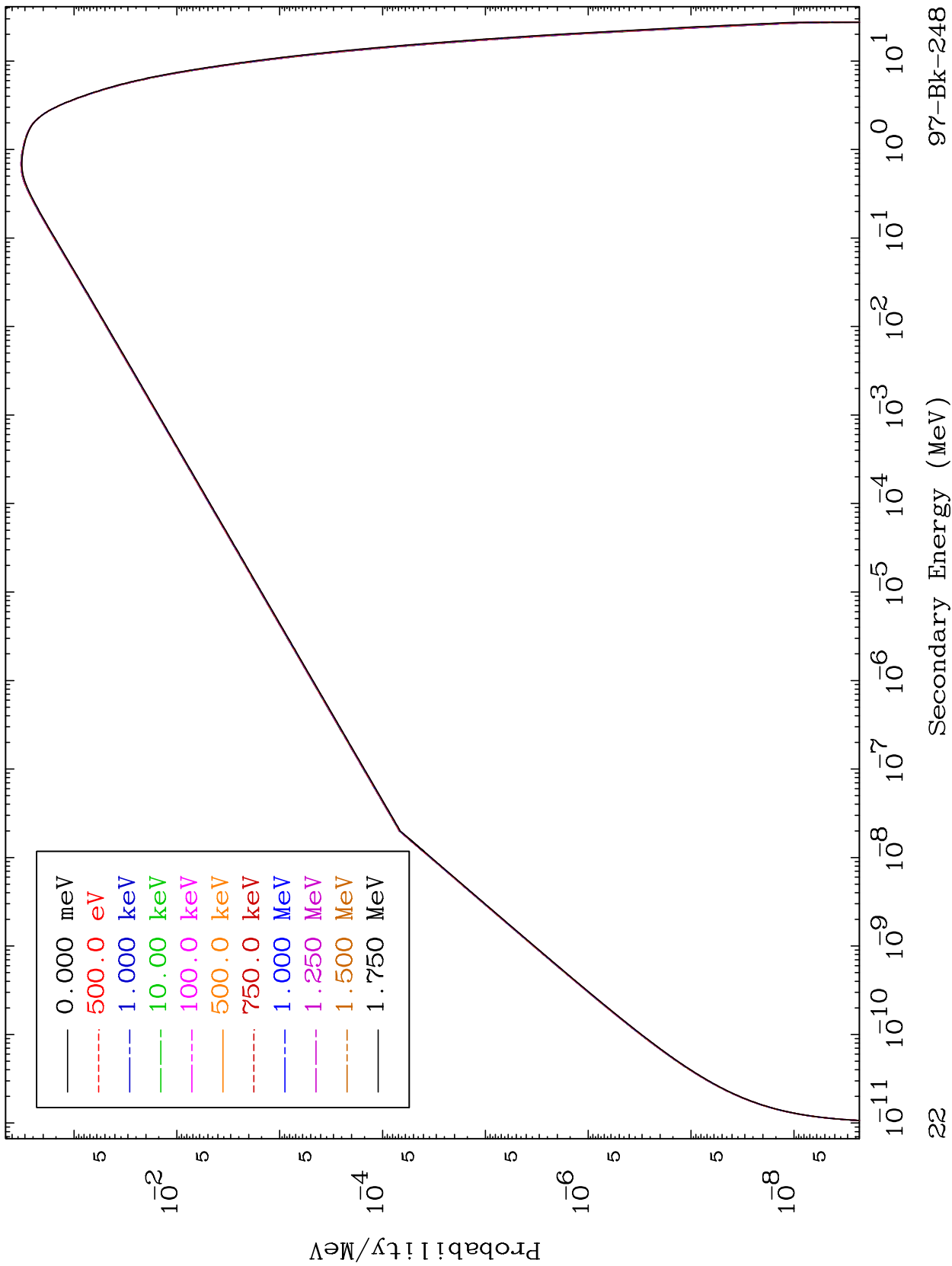
Incident Energy (MeV)

97-Bk-248

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### Fission Energy Distribution

97-Bk-248



- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

97-Bk-248

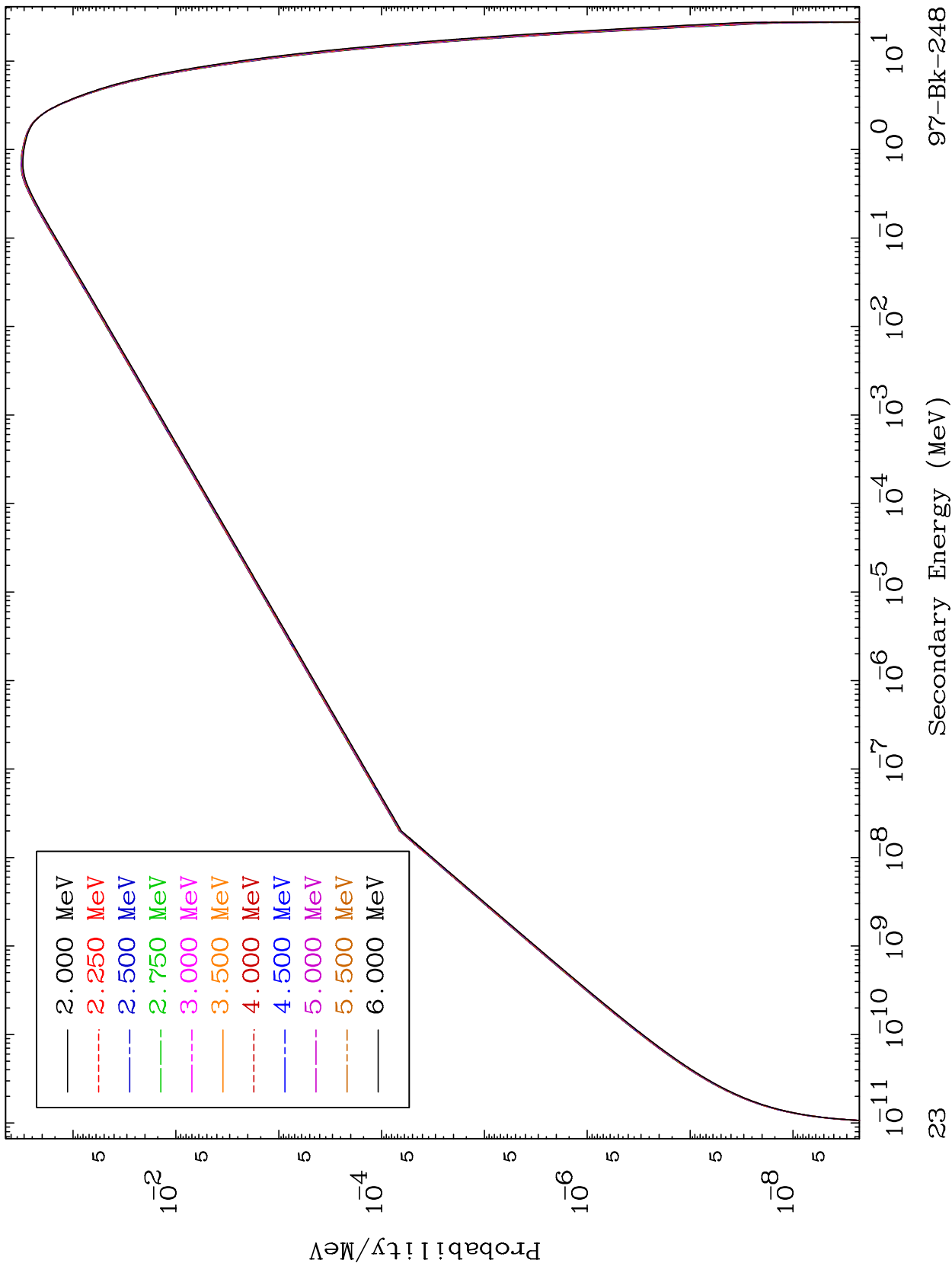
Secondary Energy (MeV)

22

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### Fission Energy Distribution

97-Bk-248



97-Bk-248

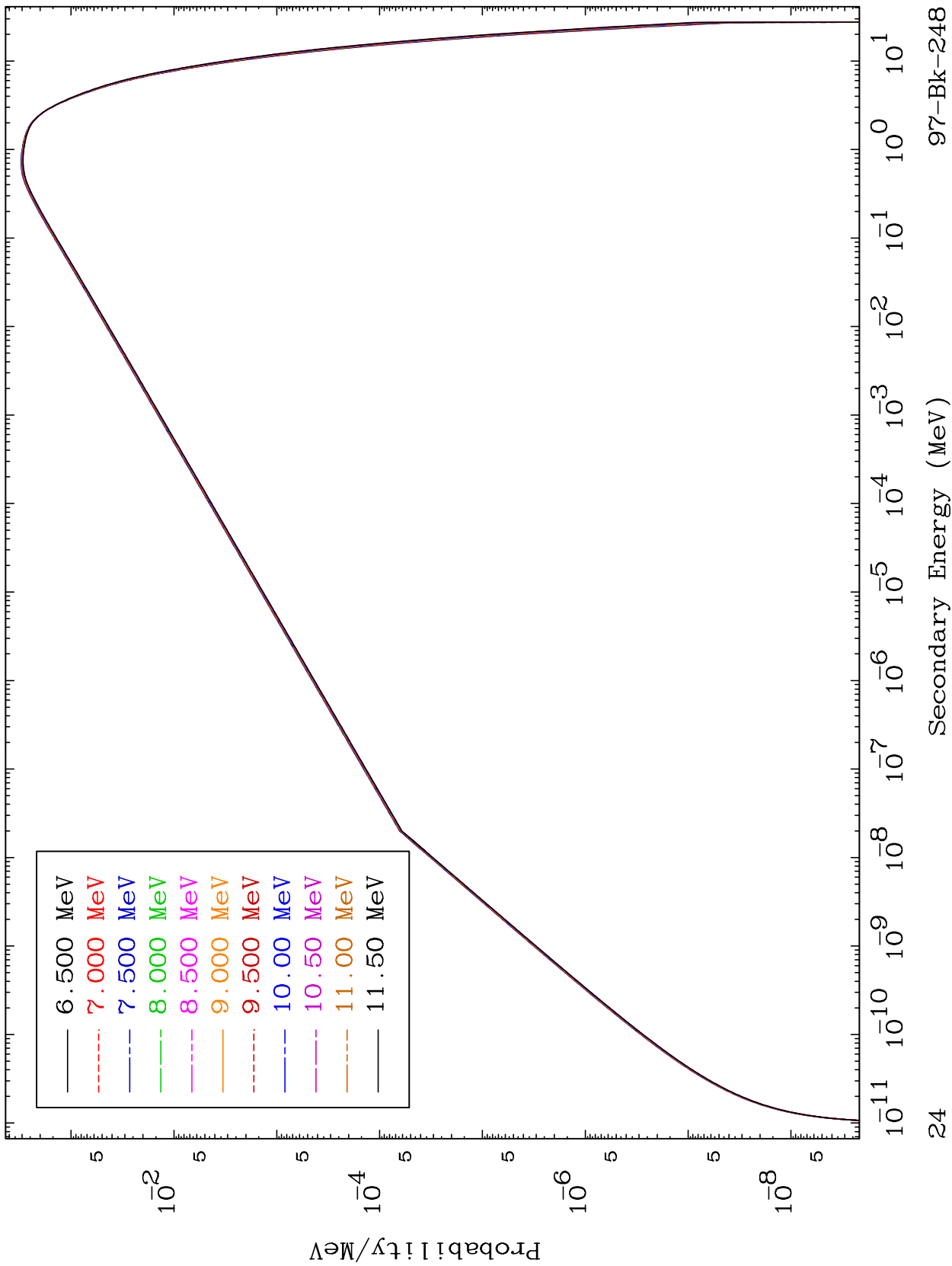
Secondary Energy (MeV)



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### Fission Energy Distribution

97-Bk-248



97-Bk-248

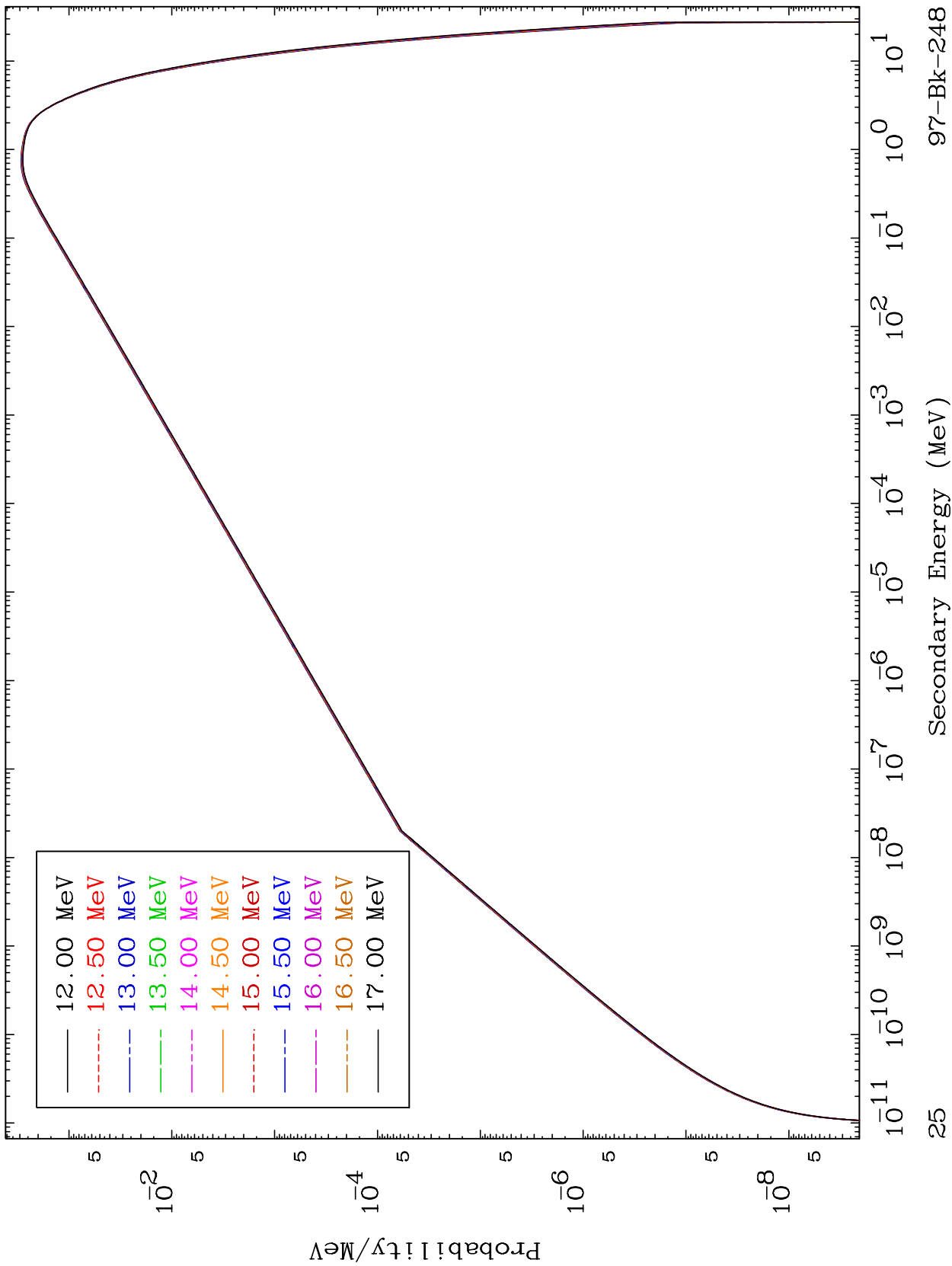
Secondary Energy (MeV)

24

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### Fission Energy Distribution

97-Bk-248



97-Bk-248

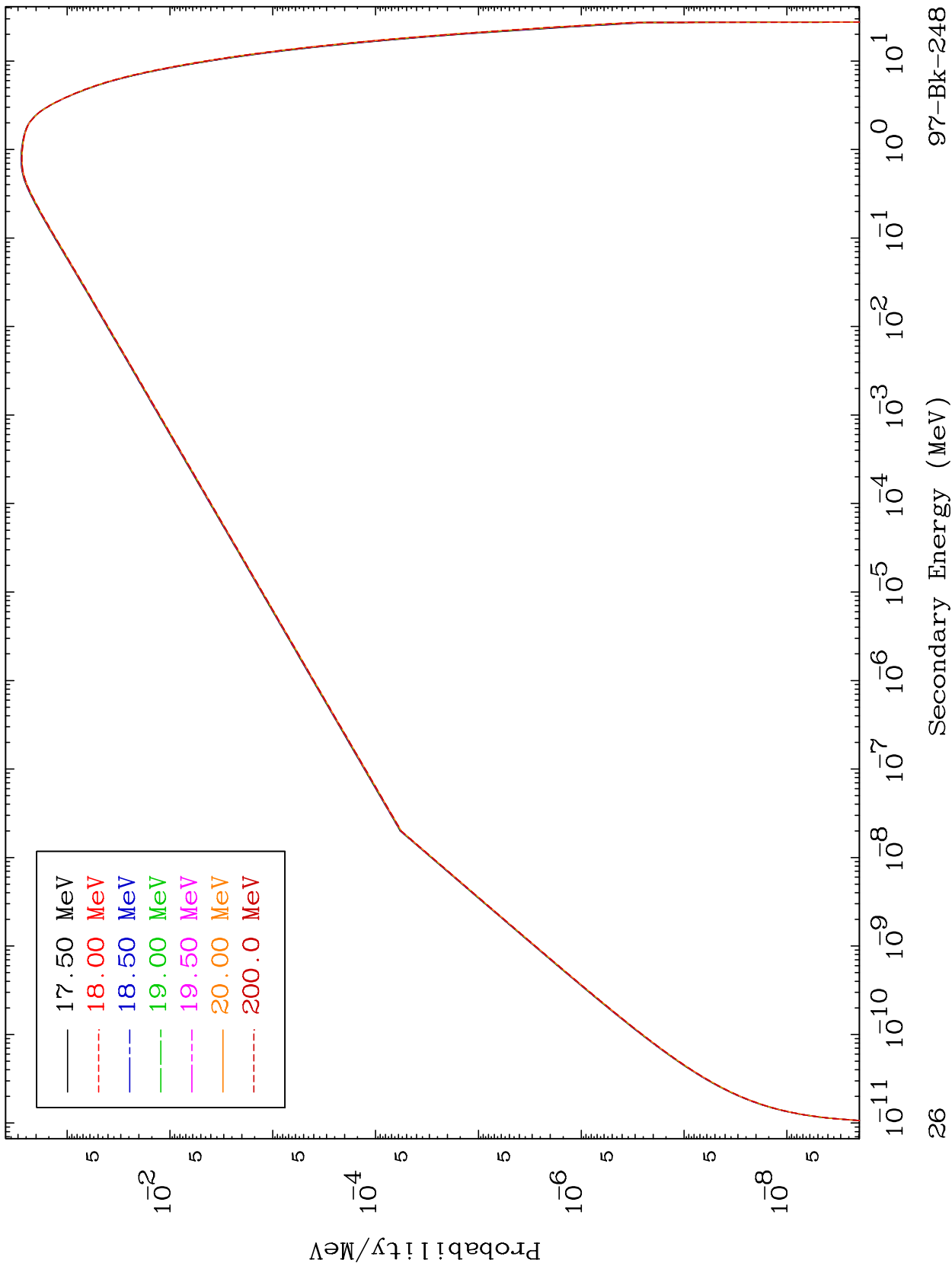
Secondary Energy (MeV)

25

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### Fission Energy Distribution

97-Bk-248



97-Bk-248

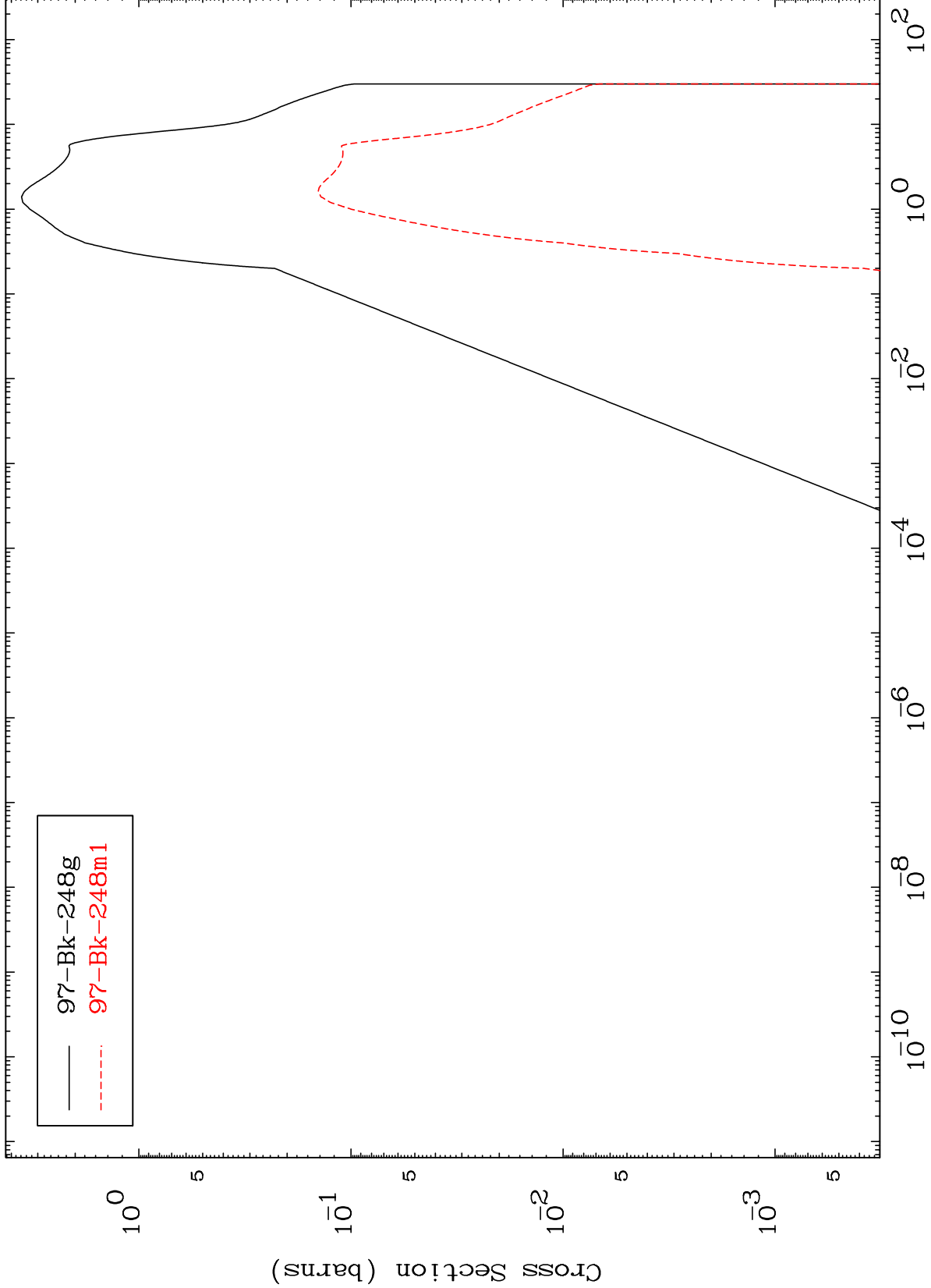
Secondary Energy (MeV)

26

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Inelastic  
Radionuclide Production Cross Section

97-Bk-248



— 97-Bk-248g  
- - - 97-Bk-248m1

Incident Energy (MeV)

97-Bk-248

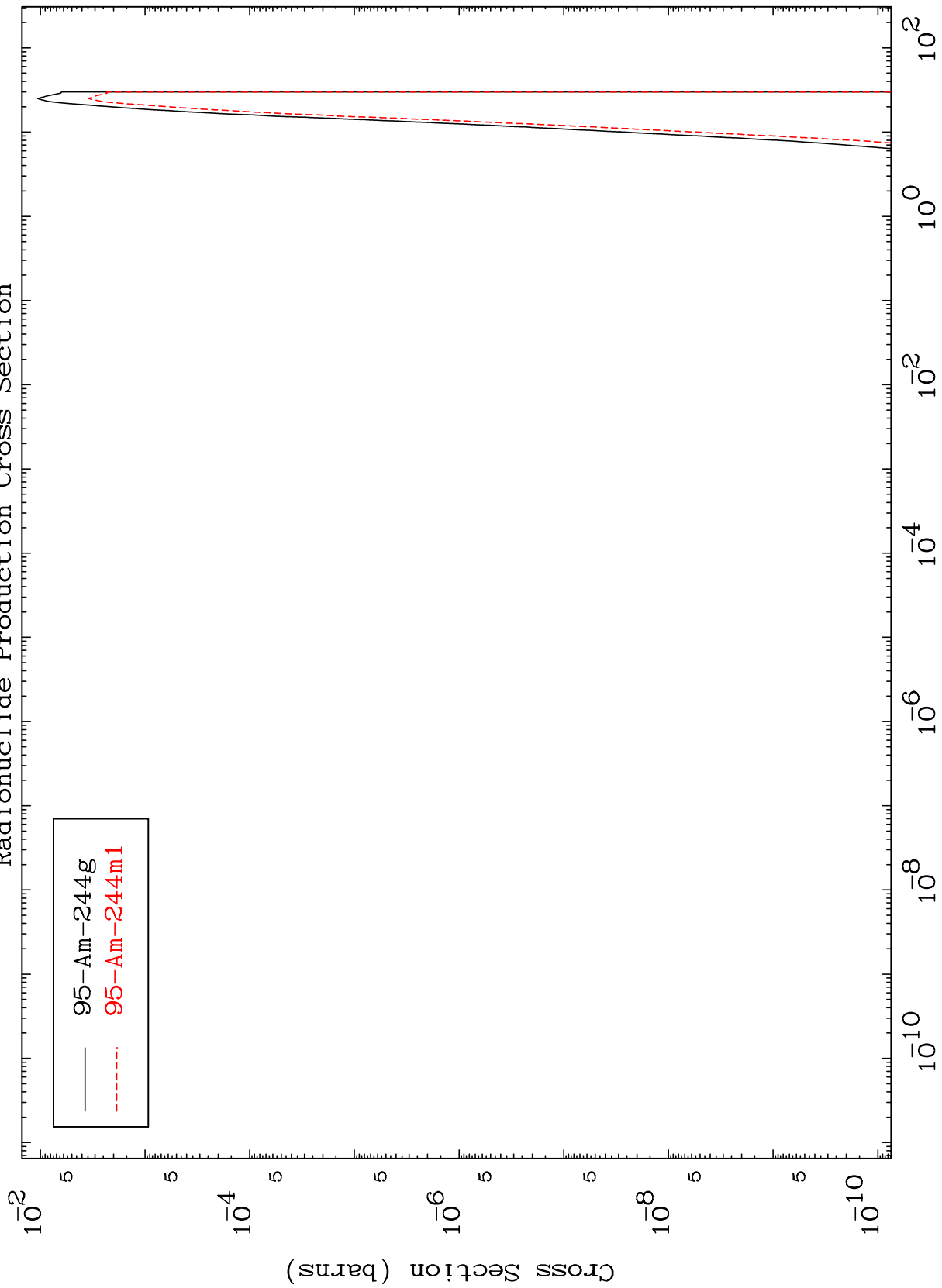
27

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(n,n')  $\alpha$

97-Bk-248

Radionuclide Production Cross Section



95-Am-244g  
95-Am-244m1

Incident Energy (MeV)

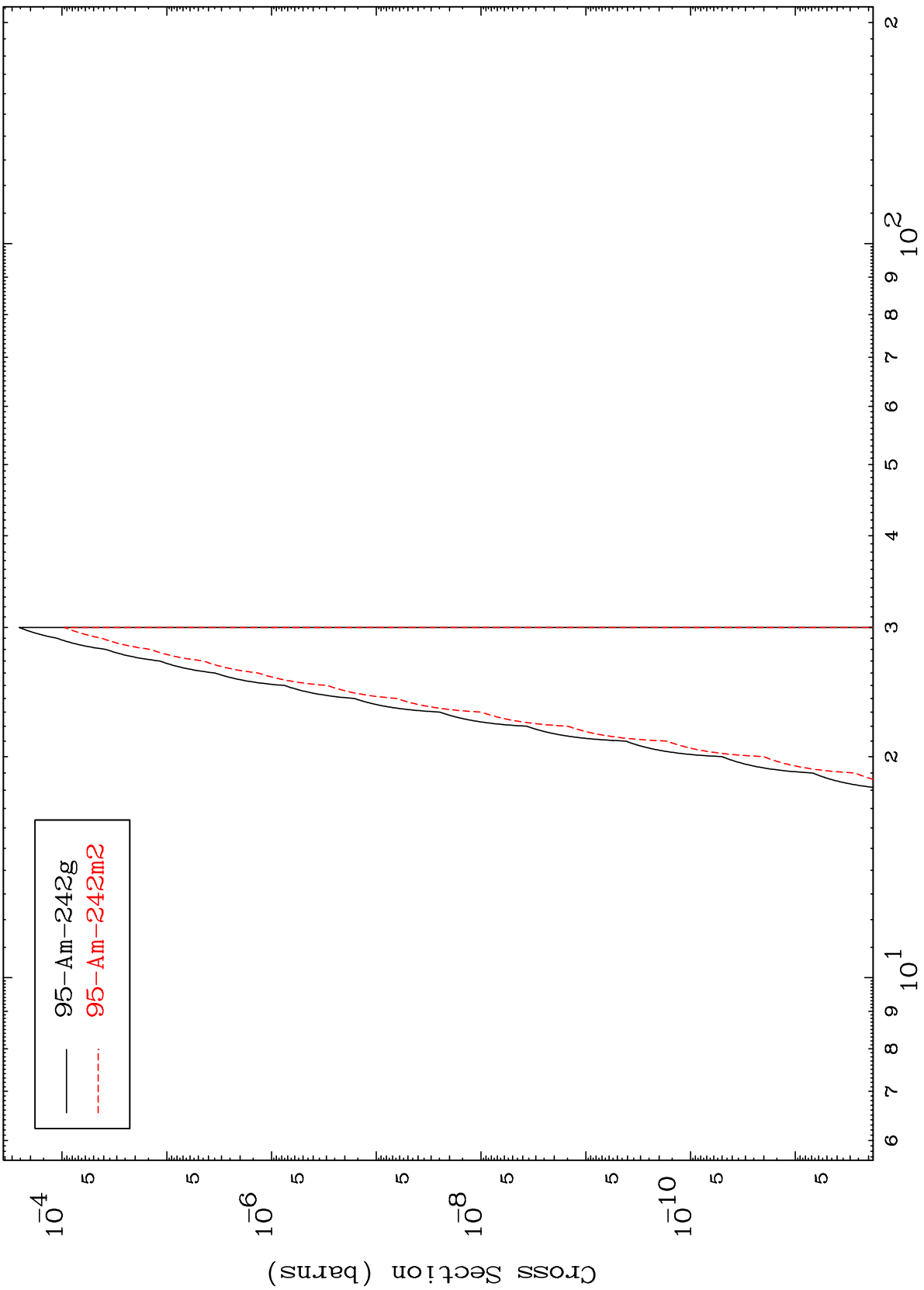
97-Bk-248

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(n,3n)  $\alpha$

97-Bk-248

Radionuclide Production Cross Section



— 95-Am-242g  
- - - 95-Am-242m2

29

Incident Energy (MeV)

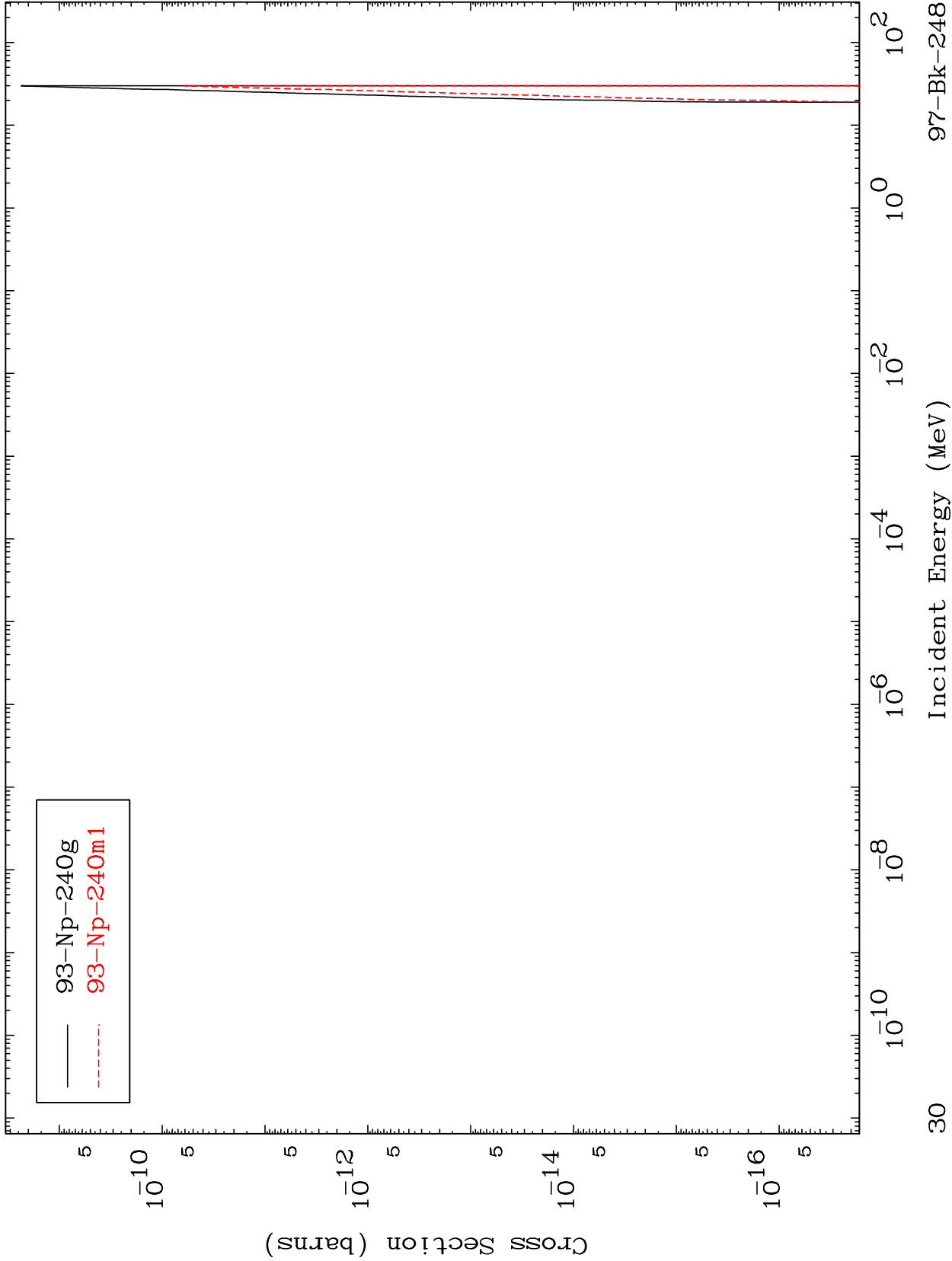
97-Bk-248

MAT 9749

(n,n') 2α

97-Bk-248

Radionuclide Production Cross Section

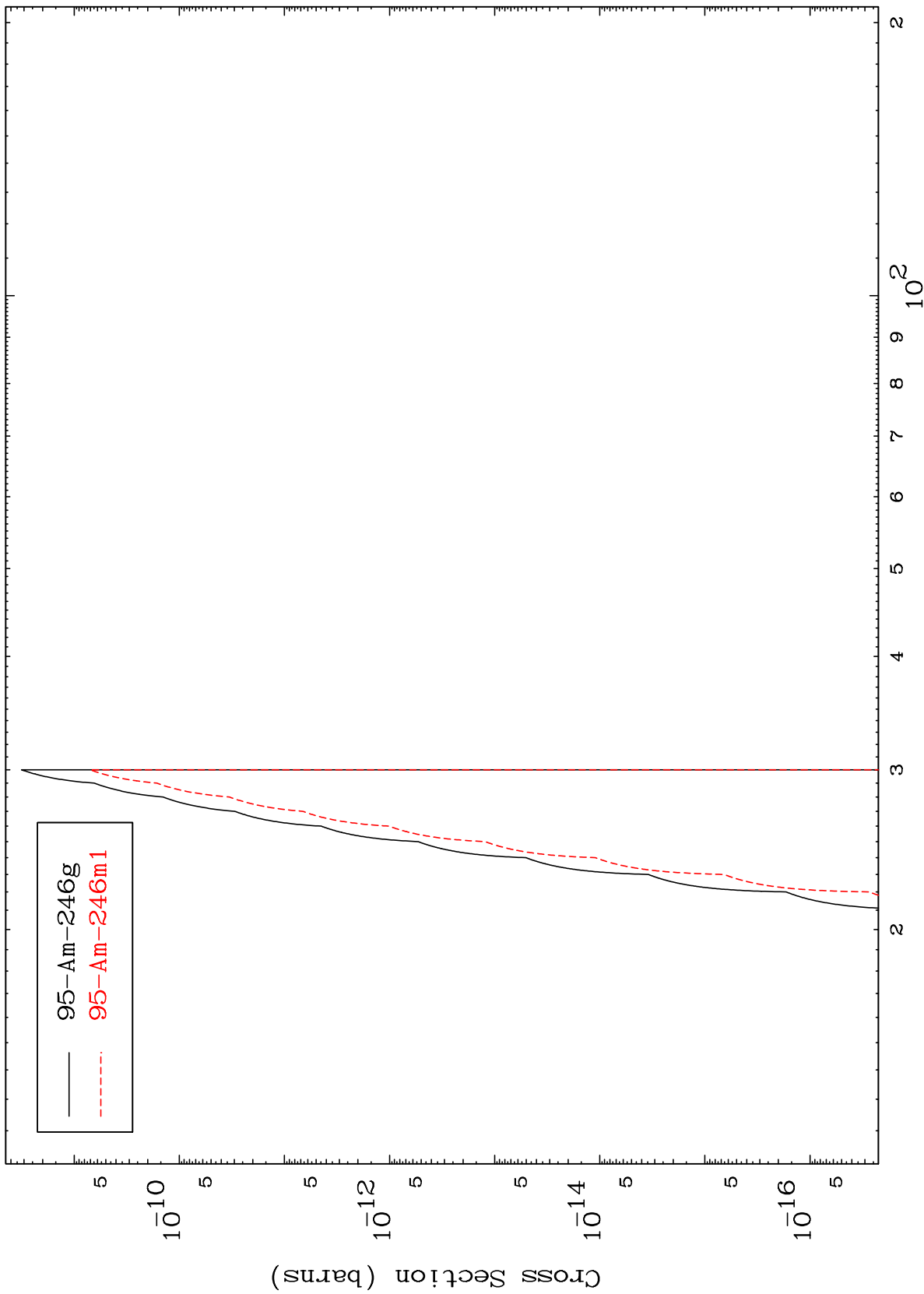


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(n,2n) p

97-Bk-248

Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-248

31

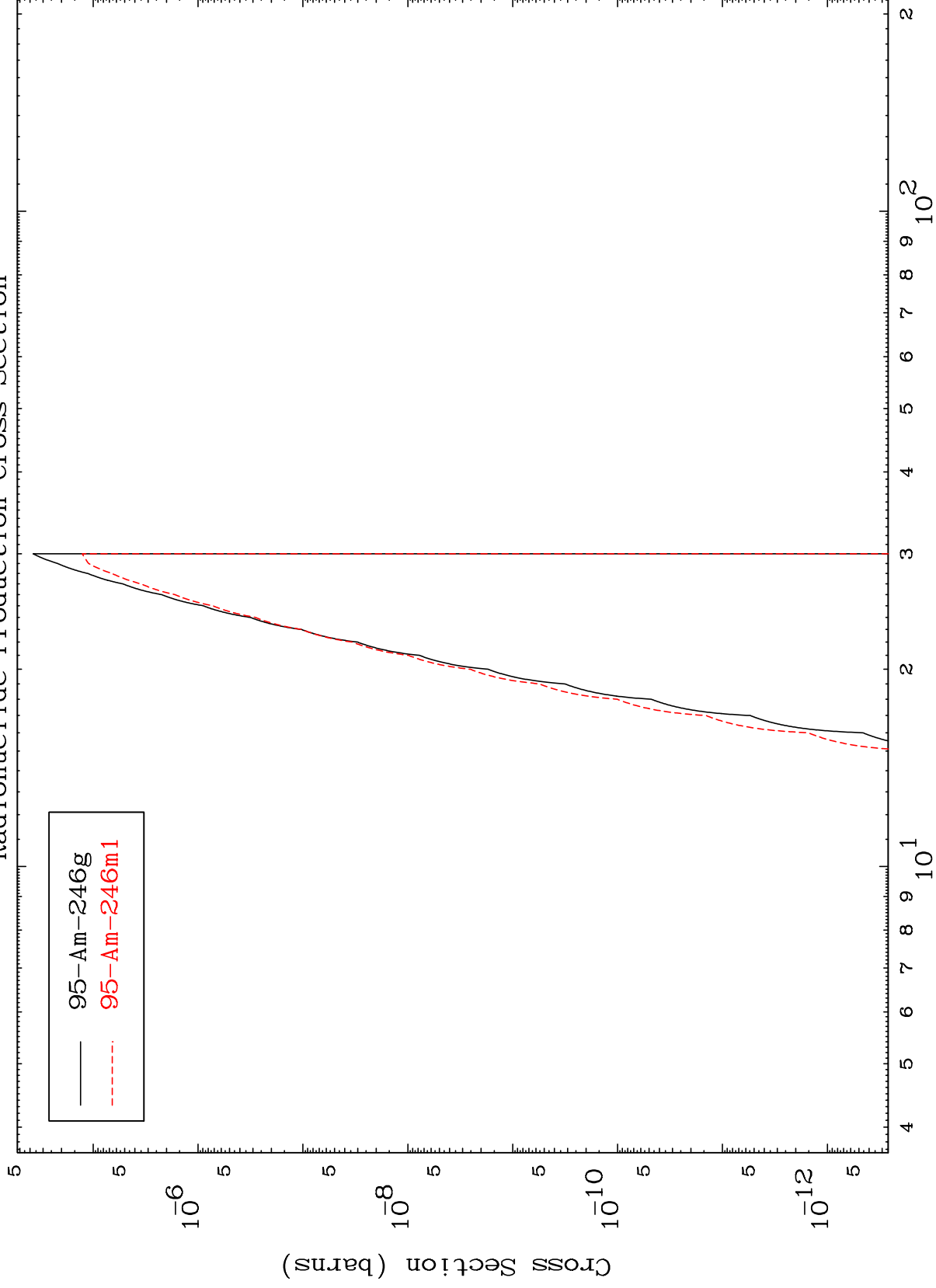


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(n,He-3)

97-Bk-248

Radionuclide Production Cross Section



32

Incident Energy (MeV)

97-Bk-248