

Program EVALPLOT  
(Version 2018-1)

by

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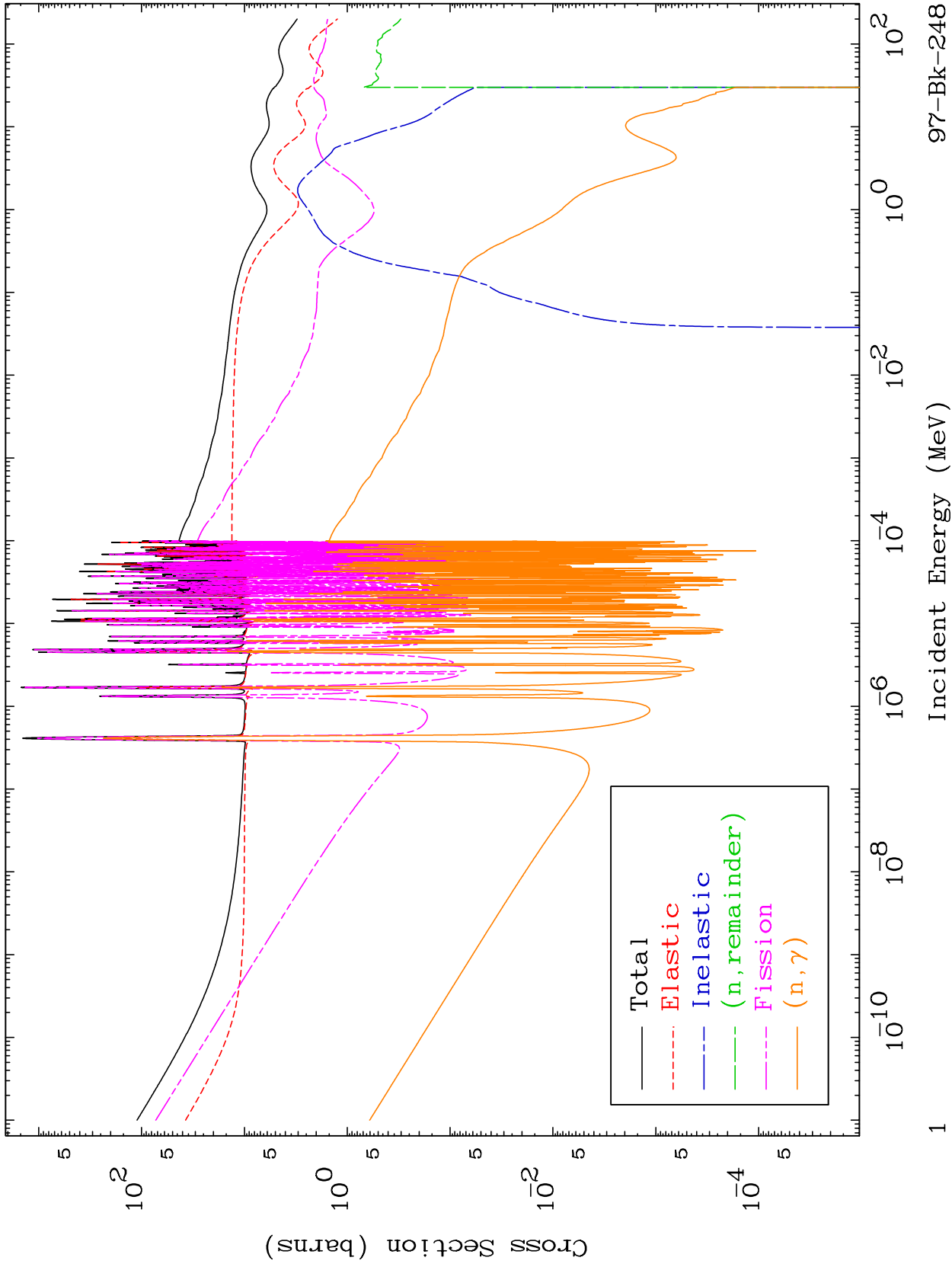
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9750

Major  
293 Kelvin Cross Sections

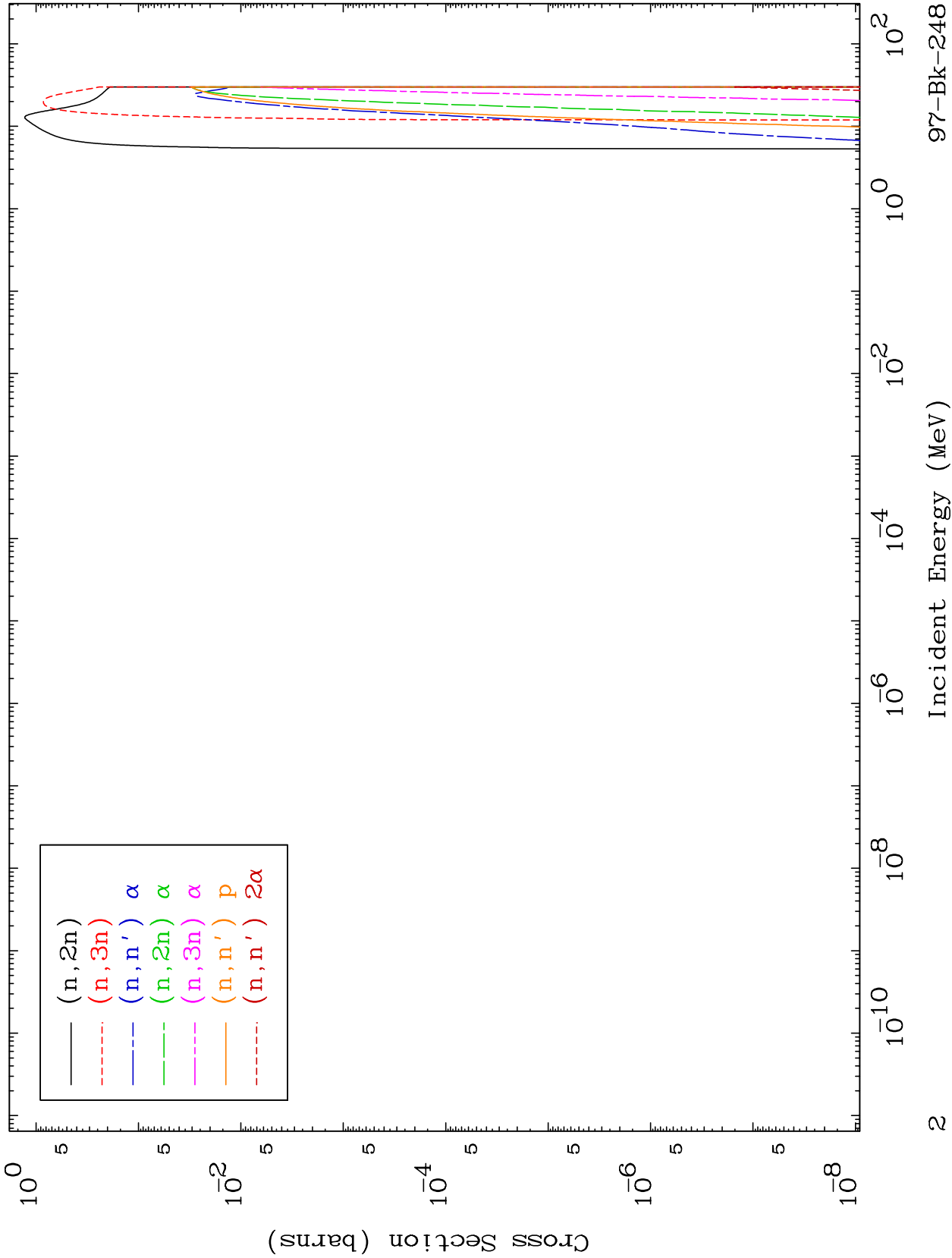
97-Bk-248



MAT 9750

Neutron Production  
293 Kelvin Cross Sections

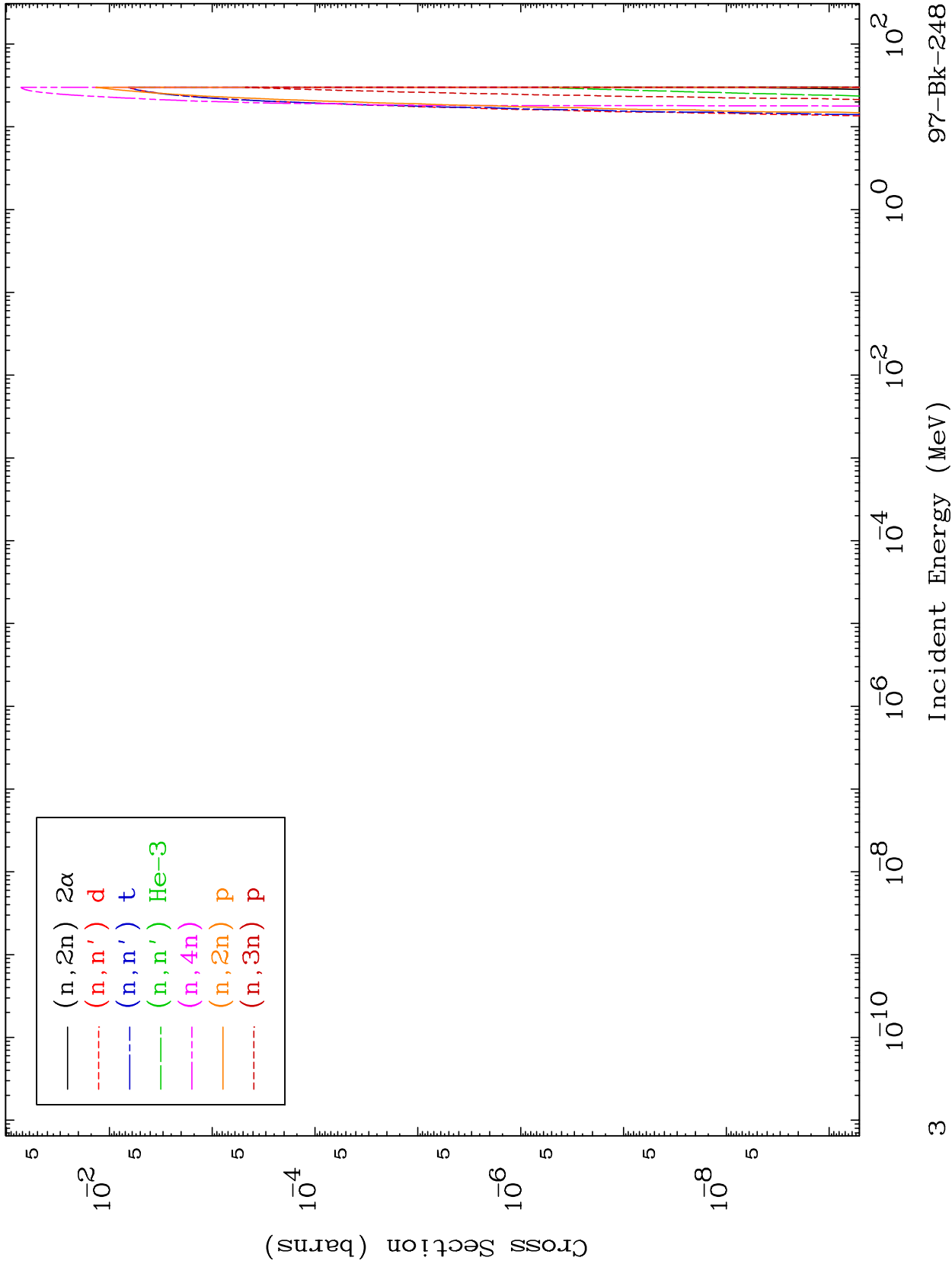
97-Bk-248



MAT 9750

Neutron Production  
293 Kelvin Cross Sections

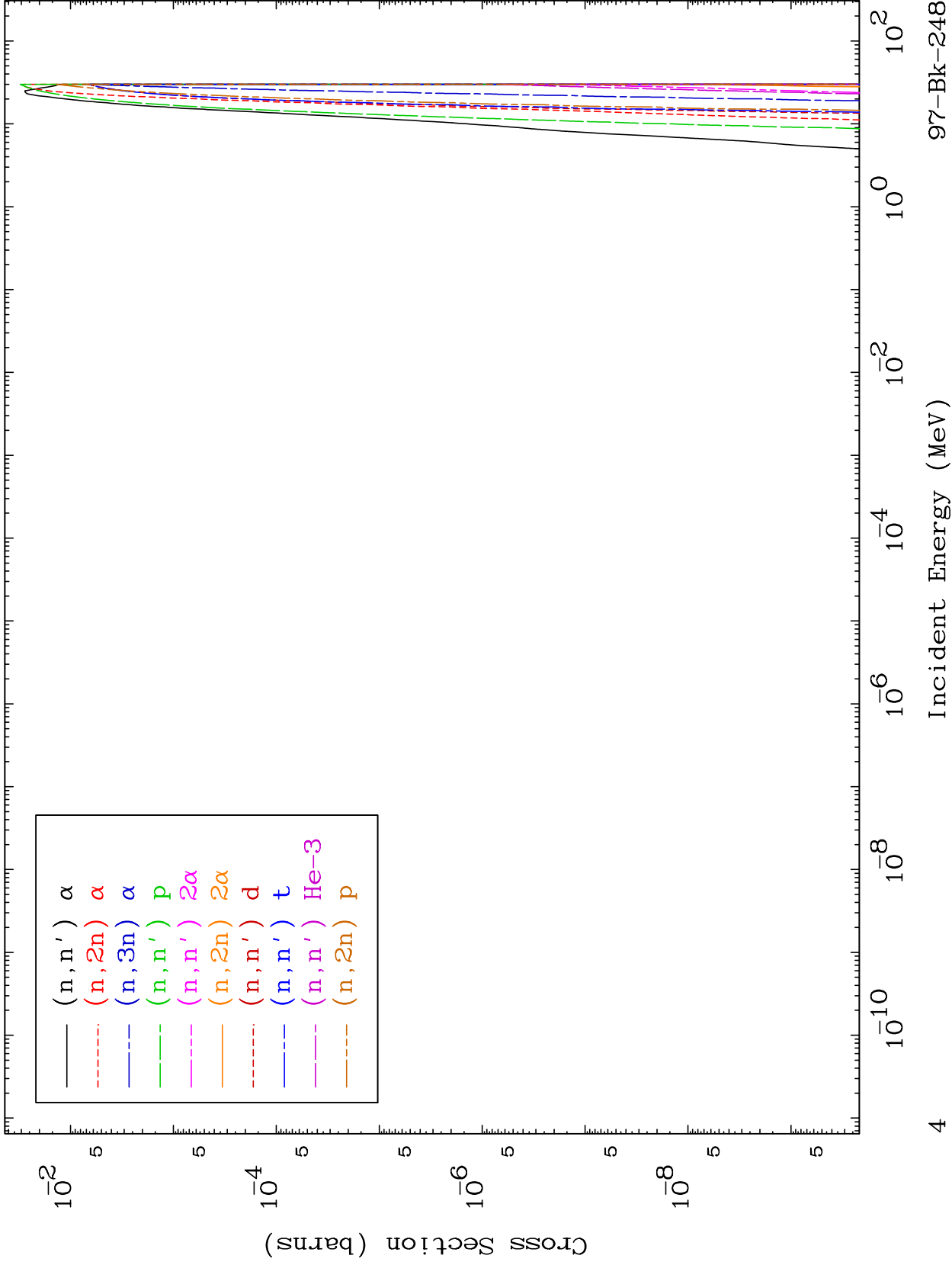
97-Bk-248



MAT 9750

Charged Particle  
293 Kelvin Cross Sections

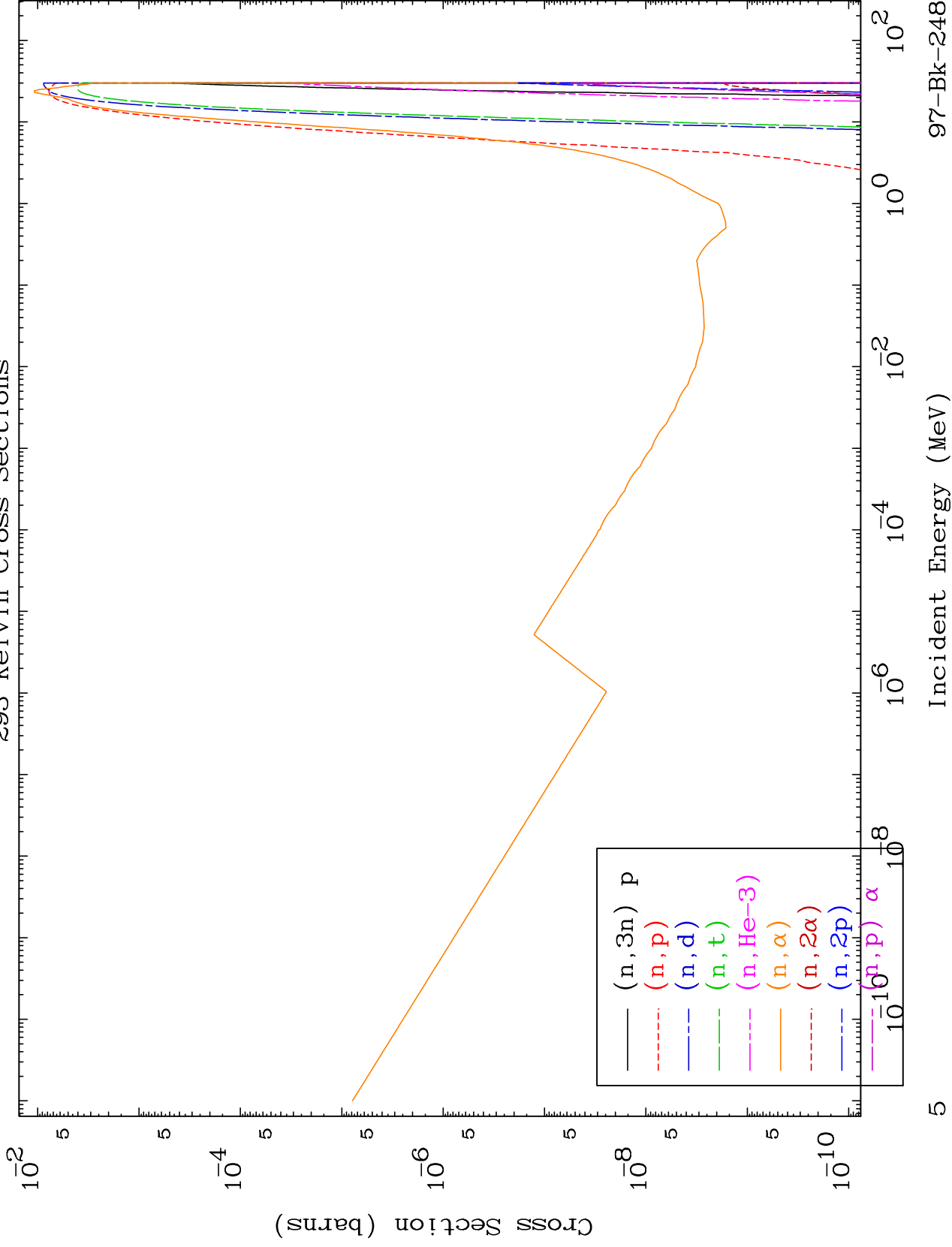
97-Bk-248



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Charged Particle  
293 Kelvin Cross Sections

97-Bk-248



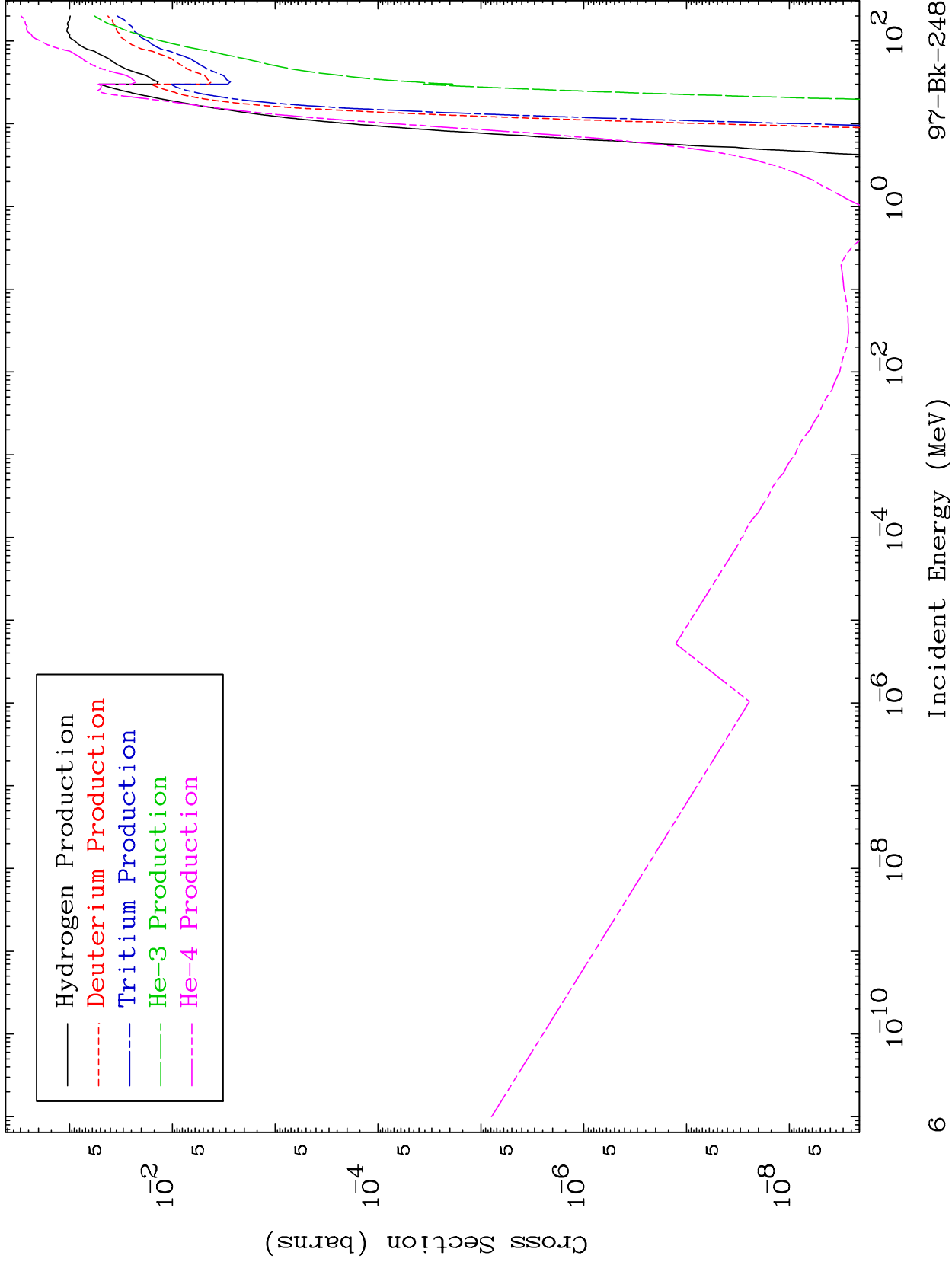
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97-Bk-248

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Particle Production  
293 Kelvin Cross Sections

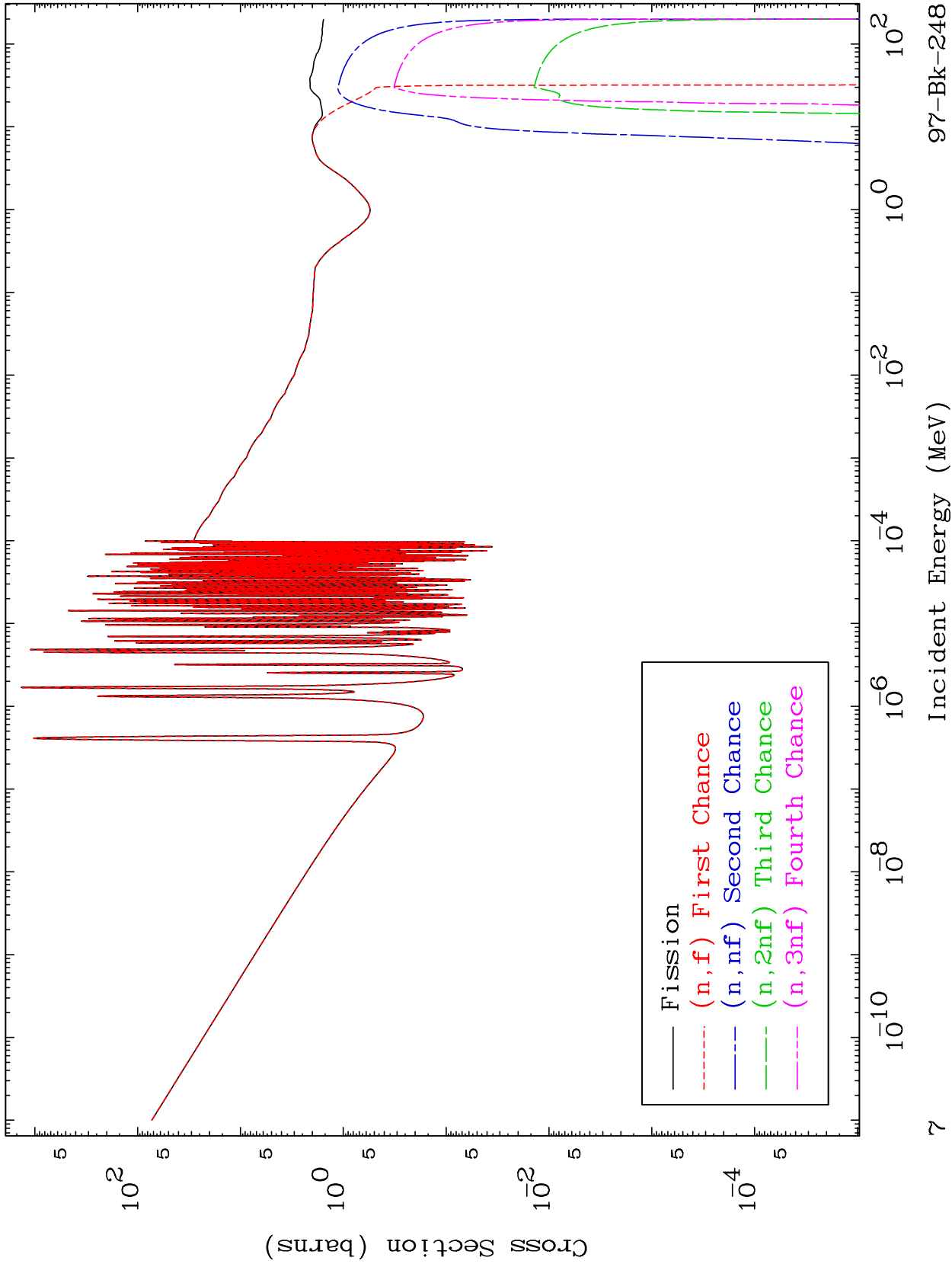
97-Bk-248



MAT 9750

Fission  
293 Kelvin Cross Sections

97-Bk-248

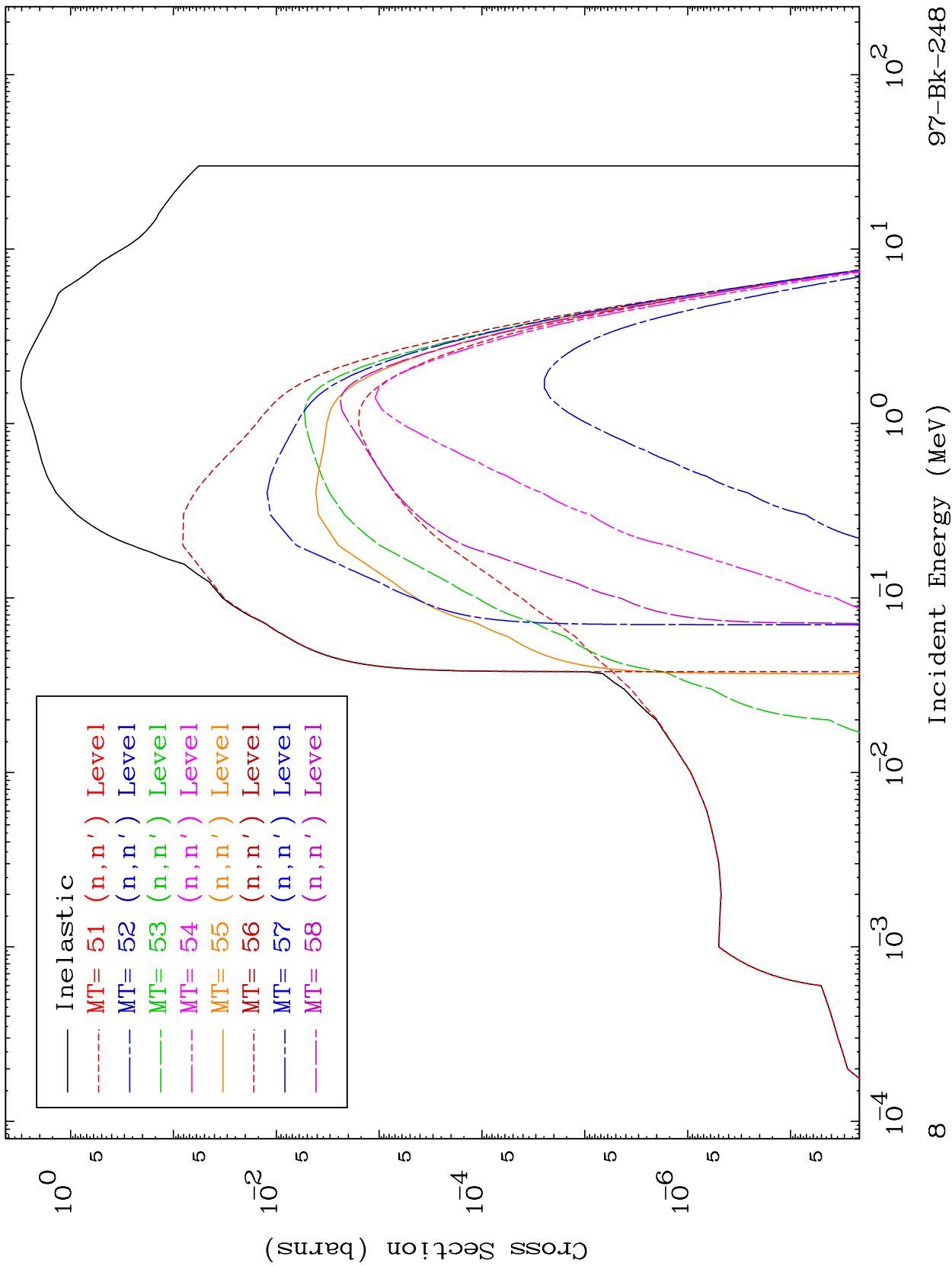




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293 Kelvin Cross Sections  
(n,n') Level

97-Bk-248

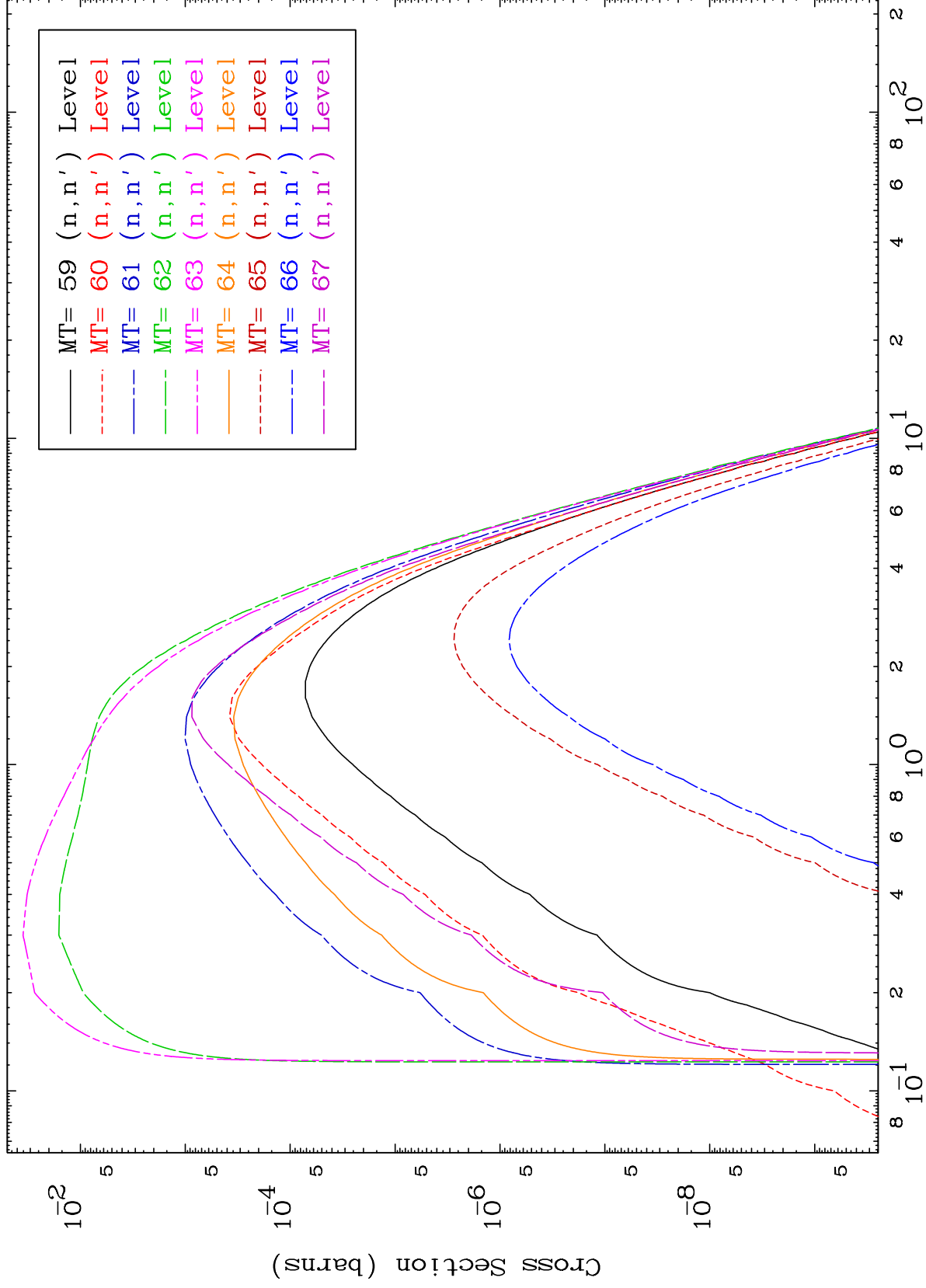


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(n,n') Level

293 Kelvin Cross Sections

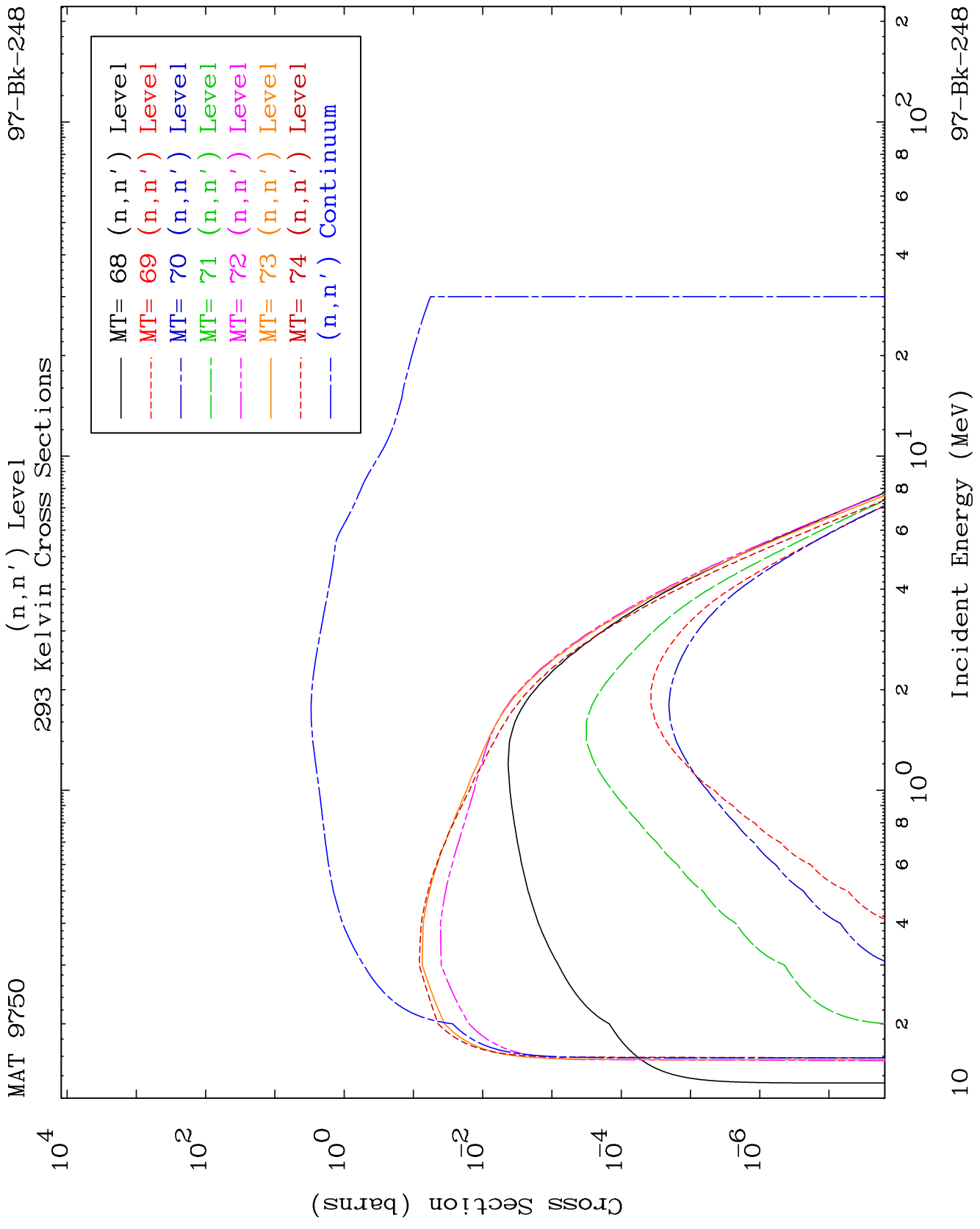
97-Bk-248

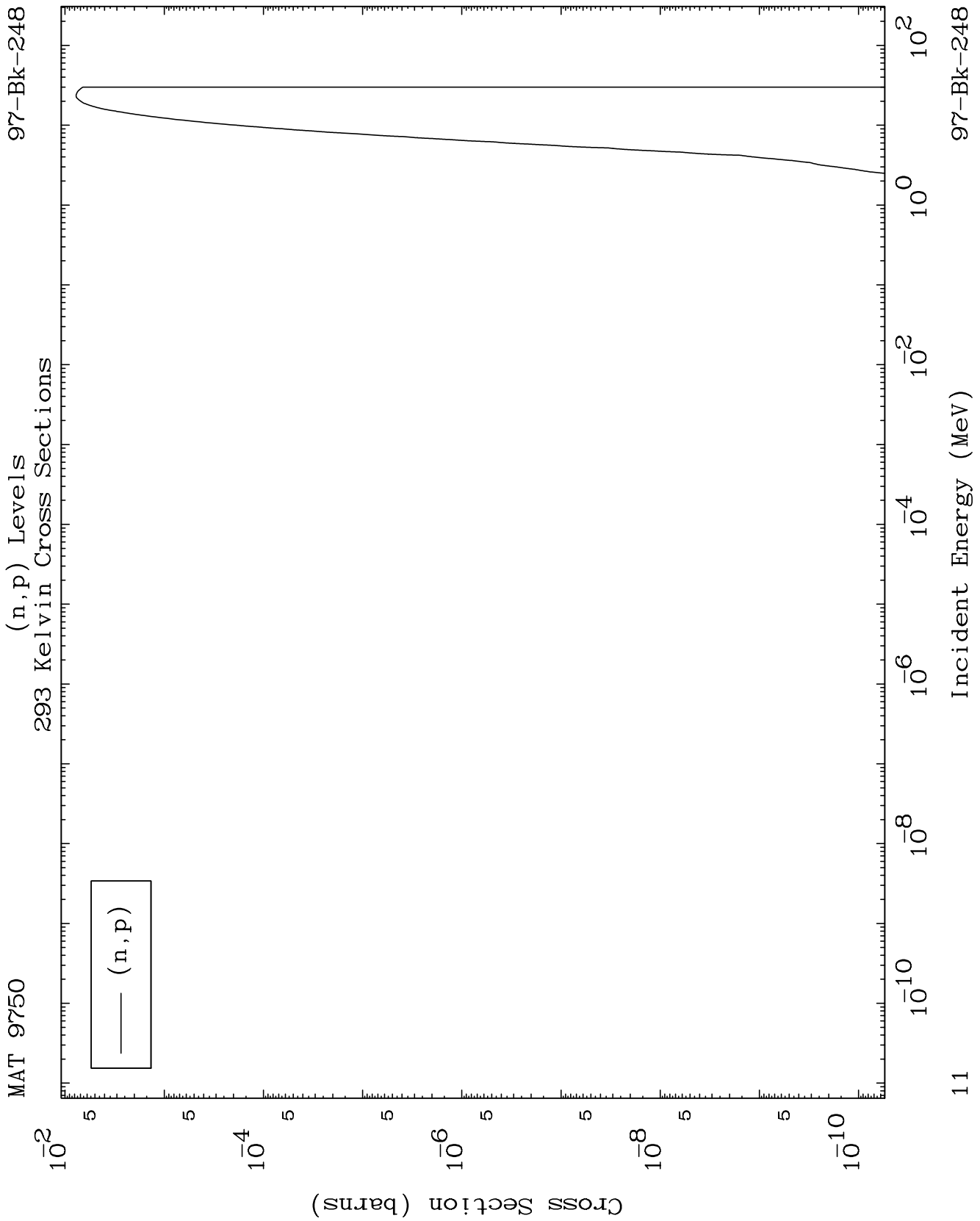


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Incident Energy (MeV)

97-Bk-248

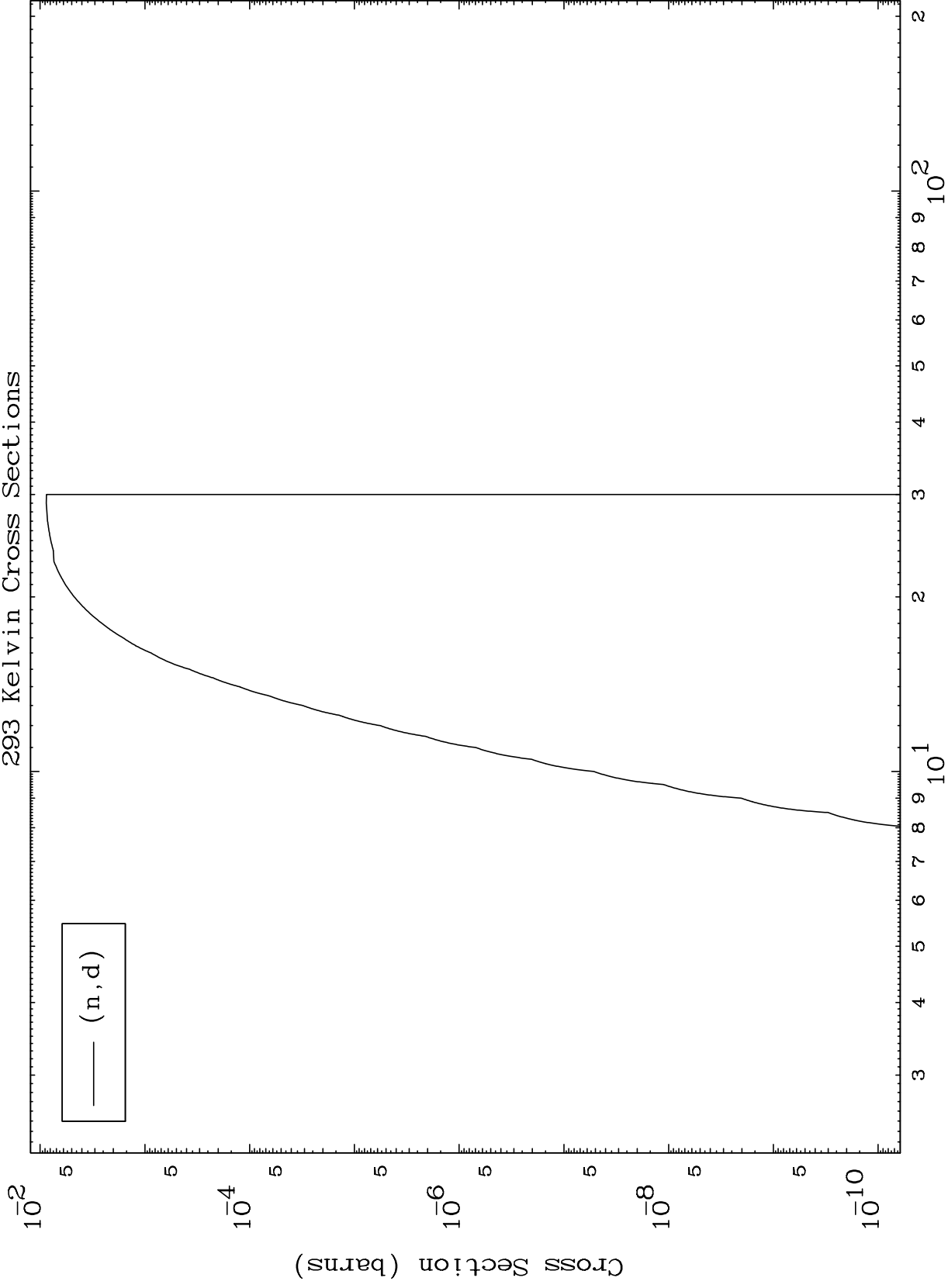




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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-248



12

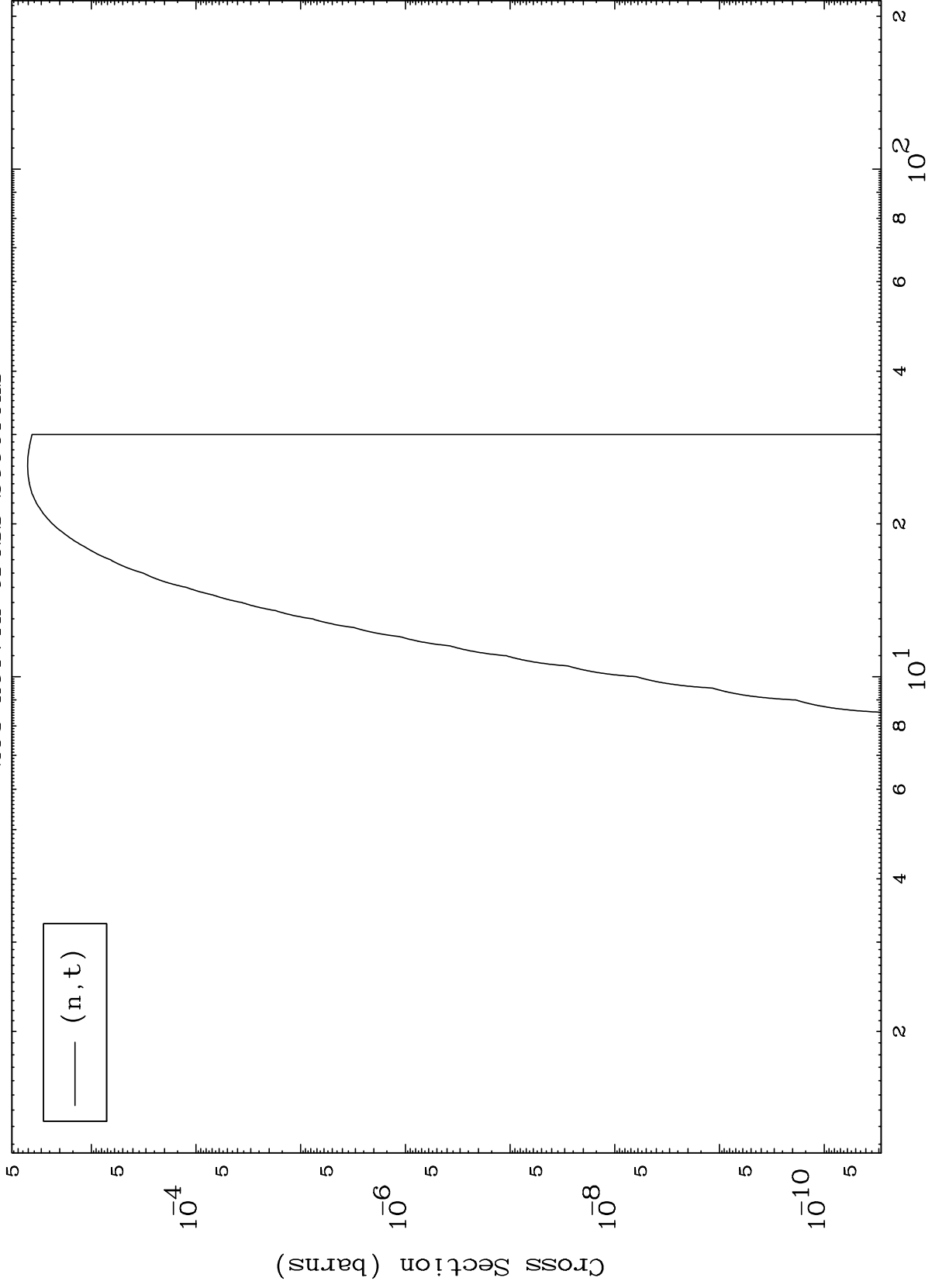
Incident Energy (MeV)

97-Bk-248

MAT 9750

(n,t) Levels  
293 Kelvin Cross Sections

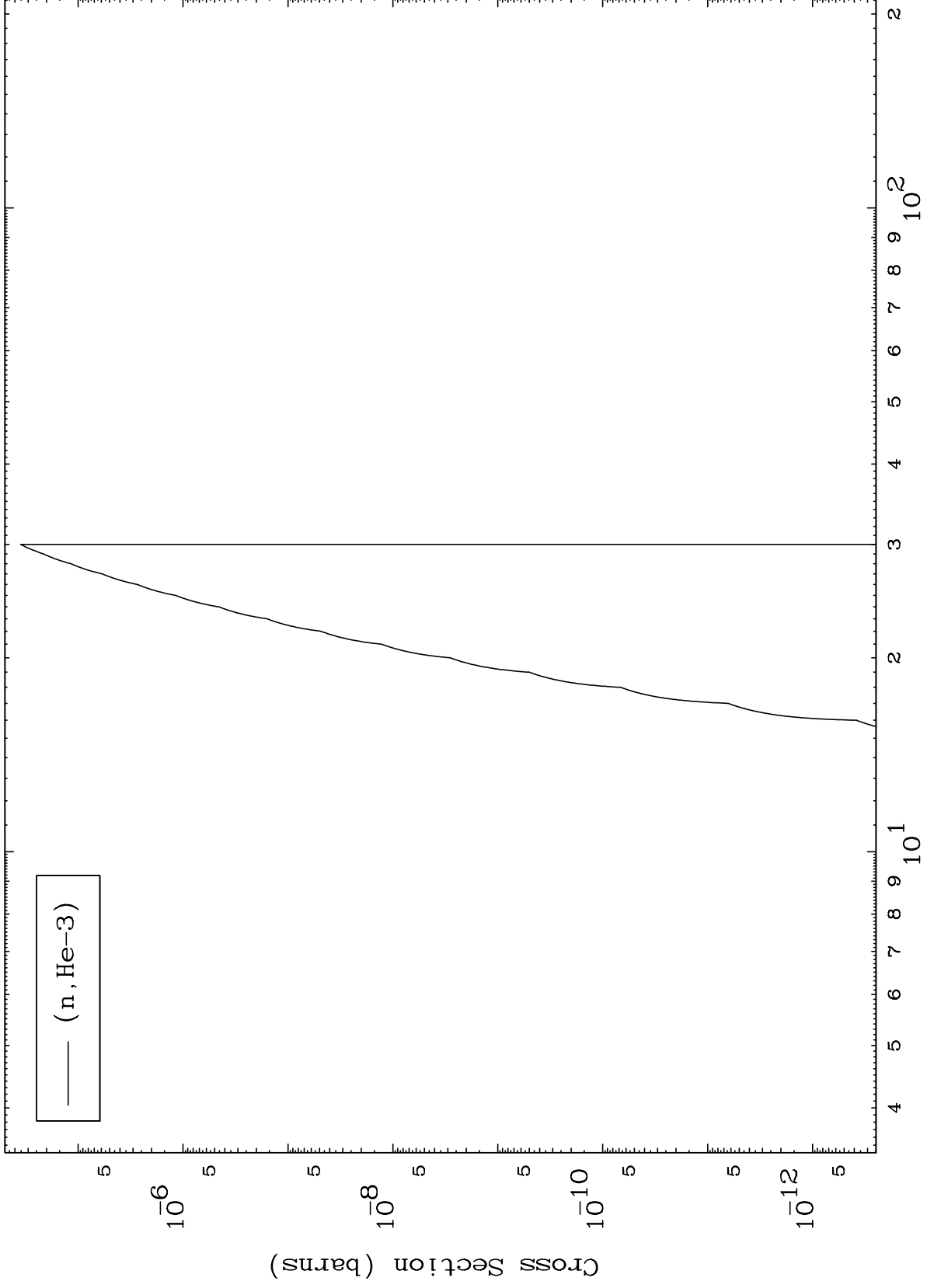
97-Bk-248



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(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-248



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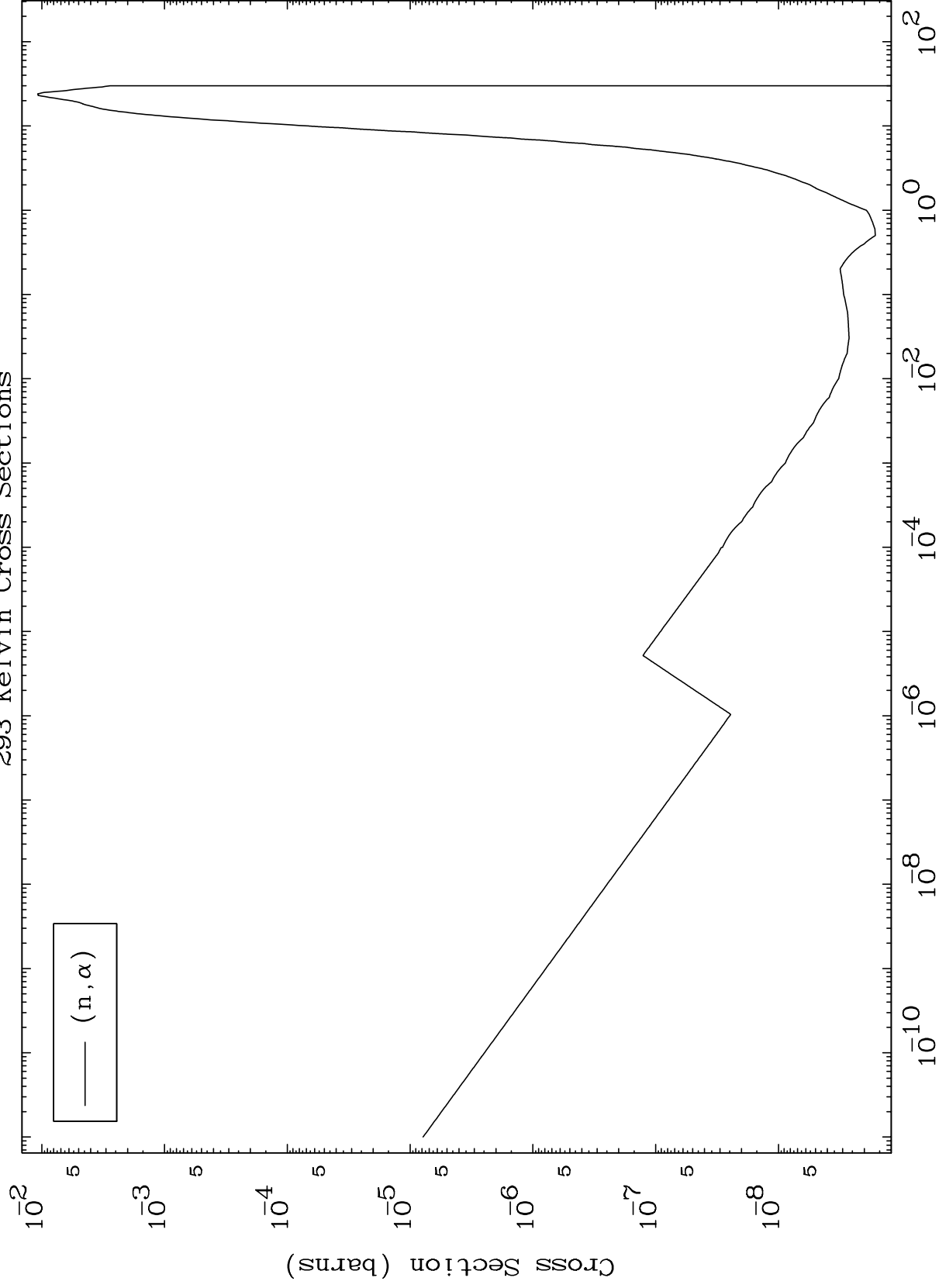
Incident Energy (MeV)

97-Bk-248

MAT 9750

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

97-Bk-248

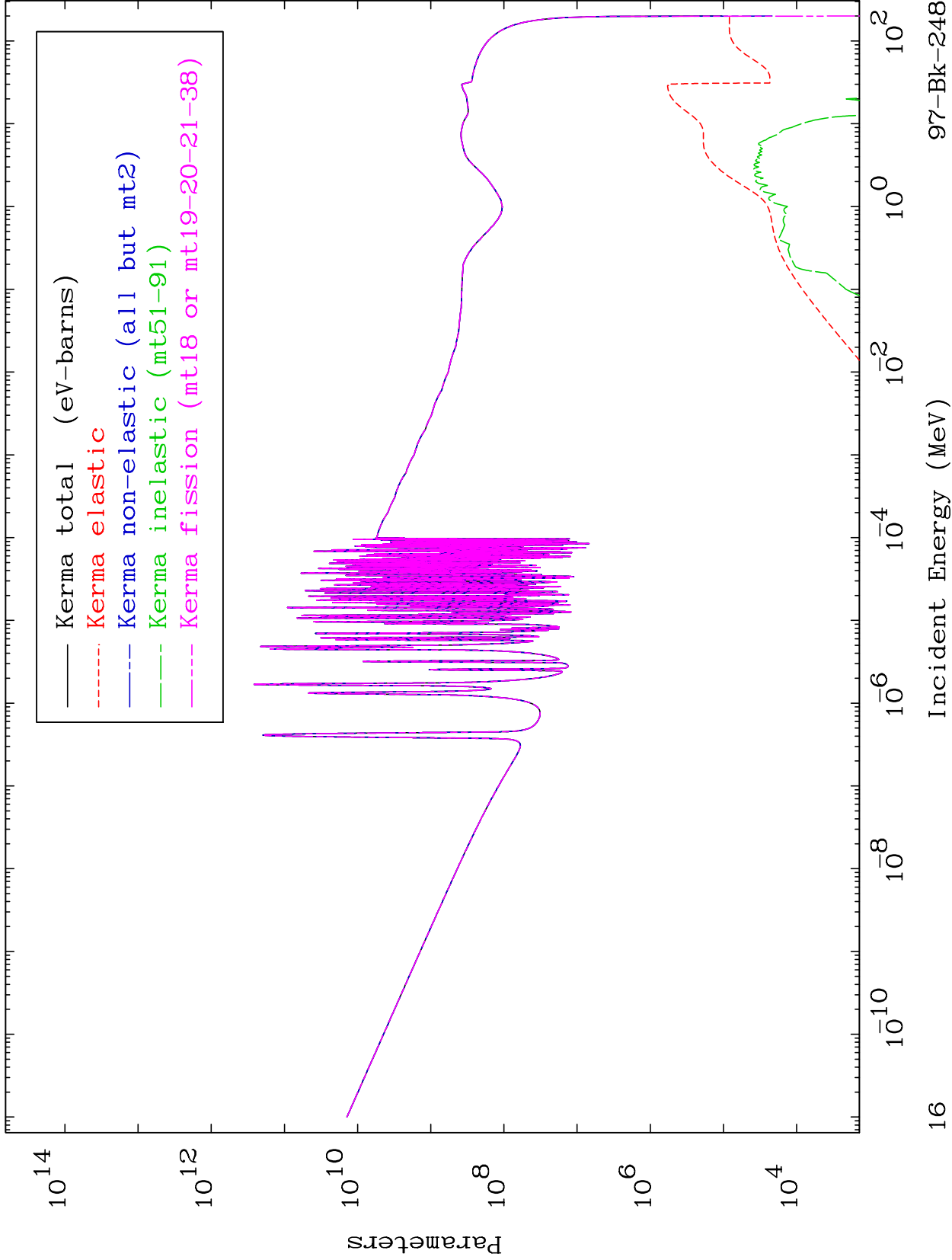


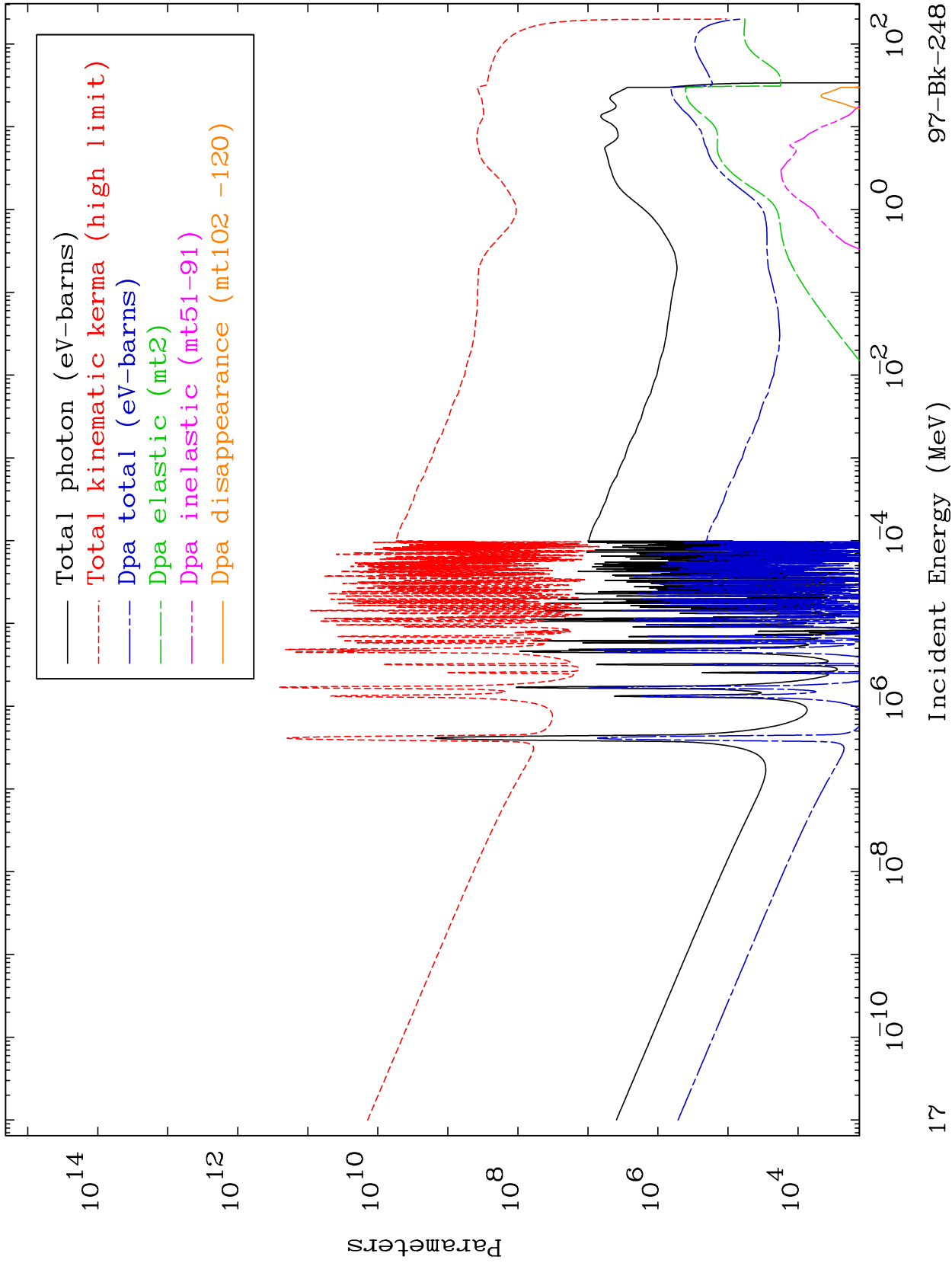
15

Incident Energy (MeV)

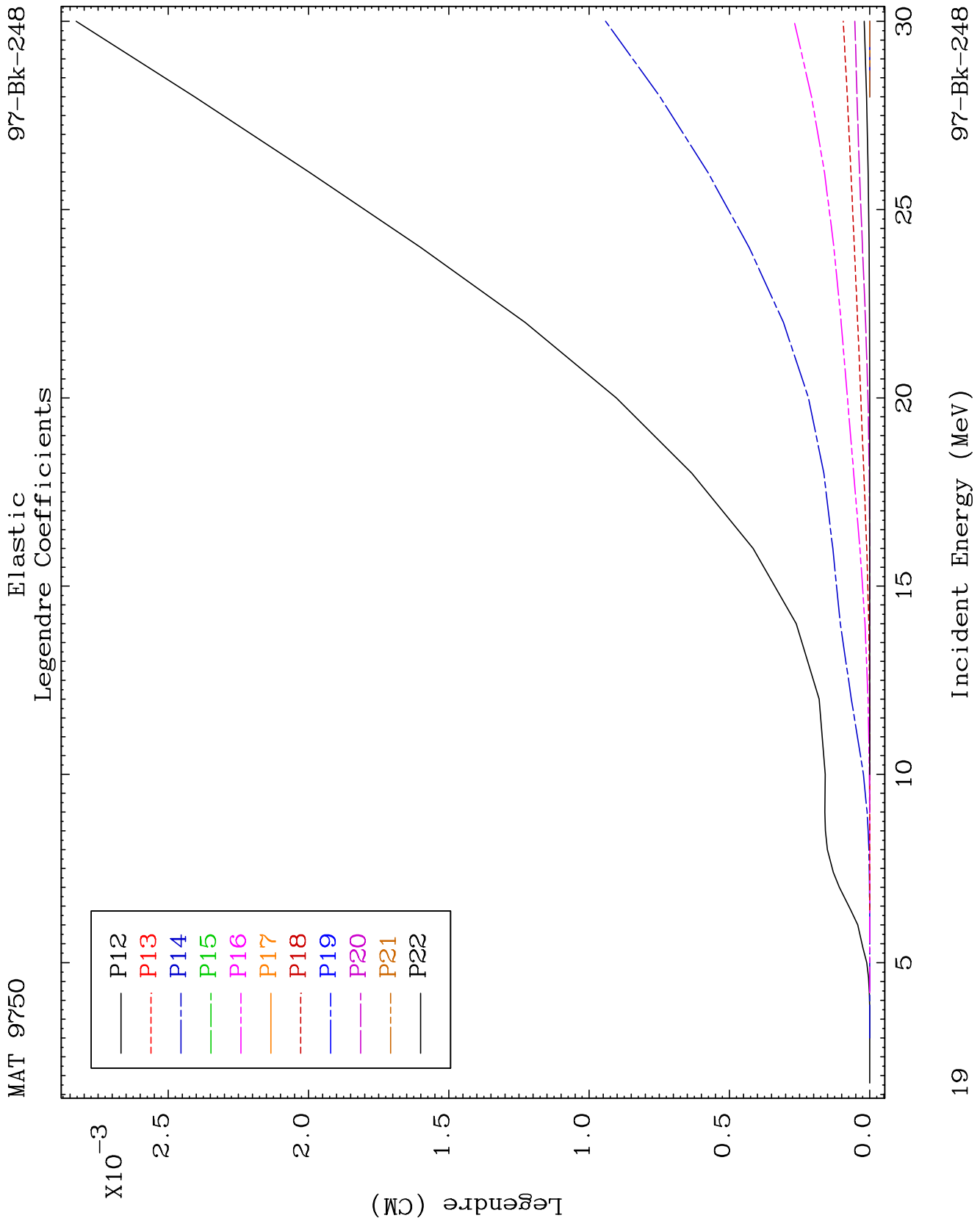
97-Bk-248









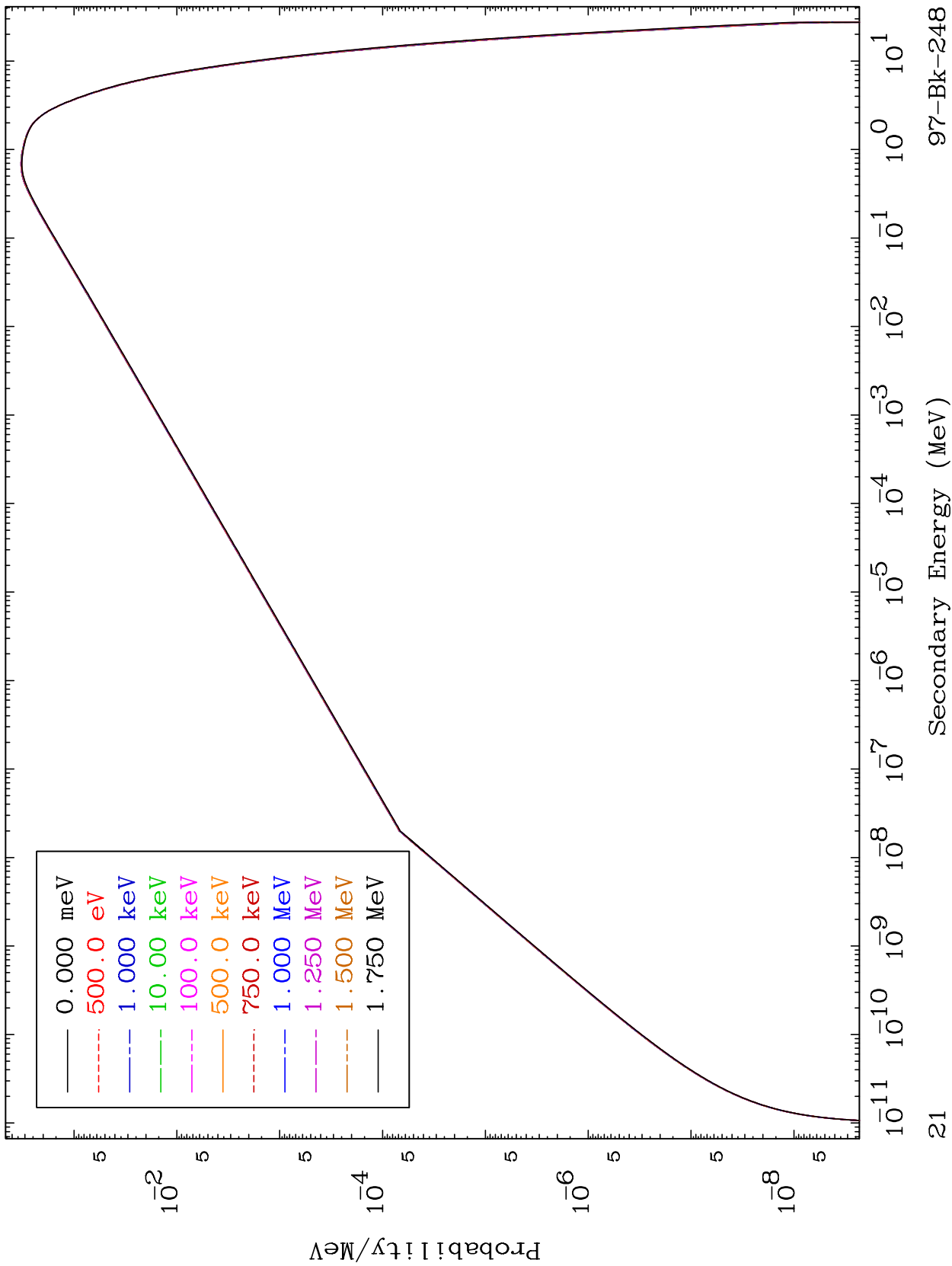




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### Fission Energy Distribution

97-Bk-248



- 0.000 meV
- 500.0 eV
- 1.000 keV
- 10.00 keV
- 100.0 keV
- 500.0 keV
- 750.0 keV
- 1.000 MeV
- 1.250 MeV
- 1.500 MeV
- 1.750 MeV

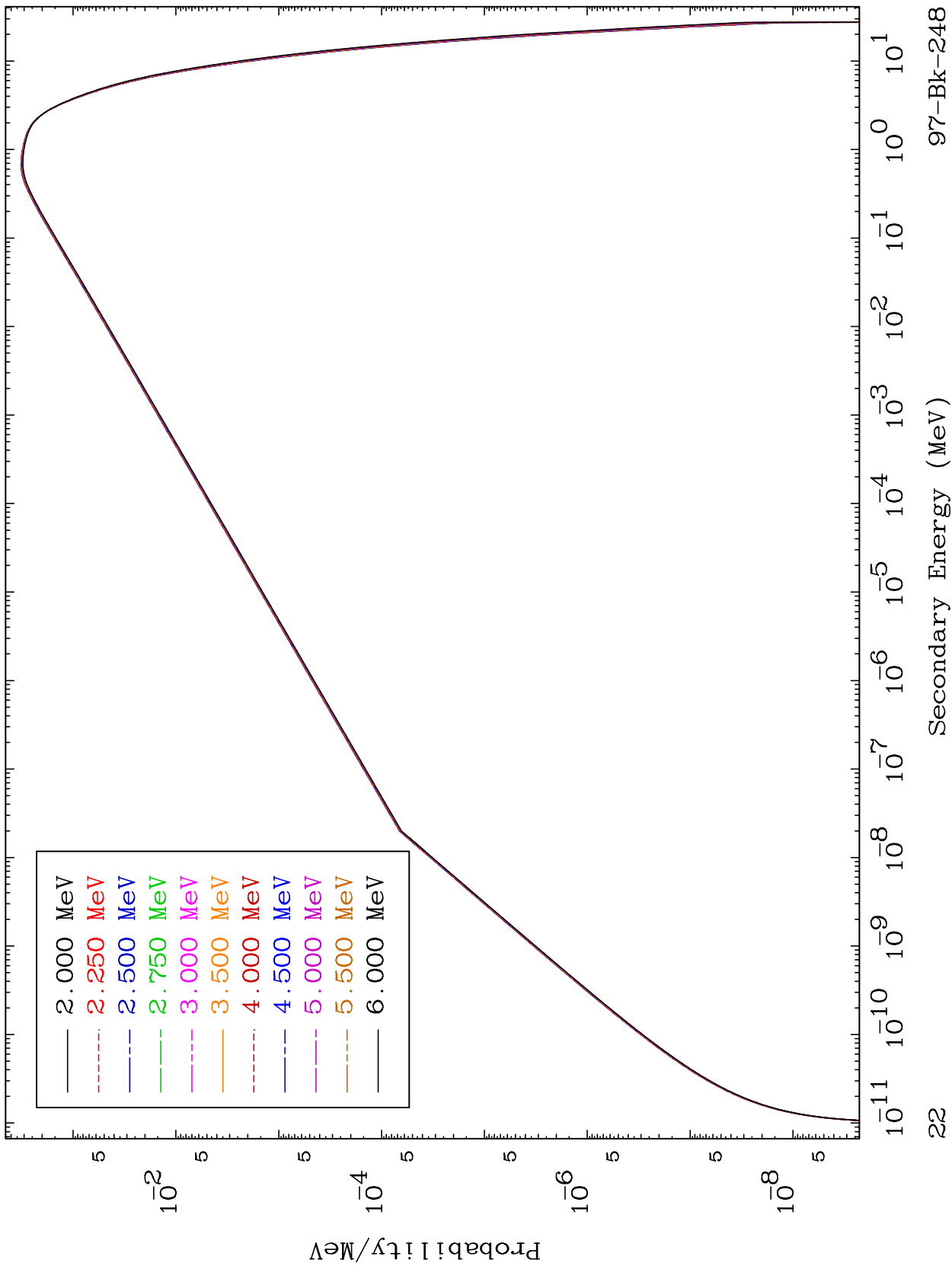
97-Bk-248

Secondary Energy (MeV)

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Fission Energy Distribution

97-Bk-248



97-Bk-248

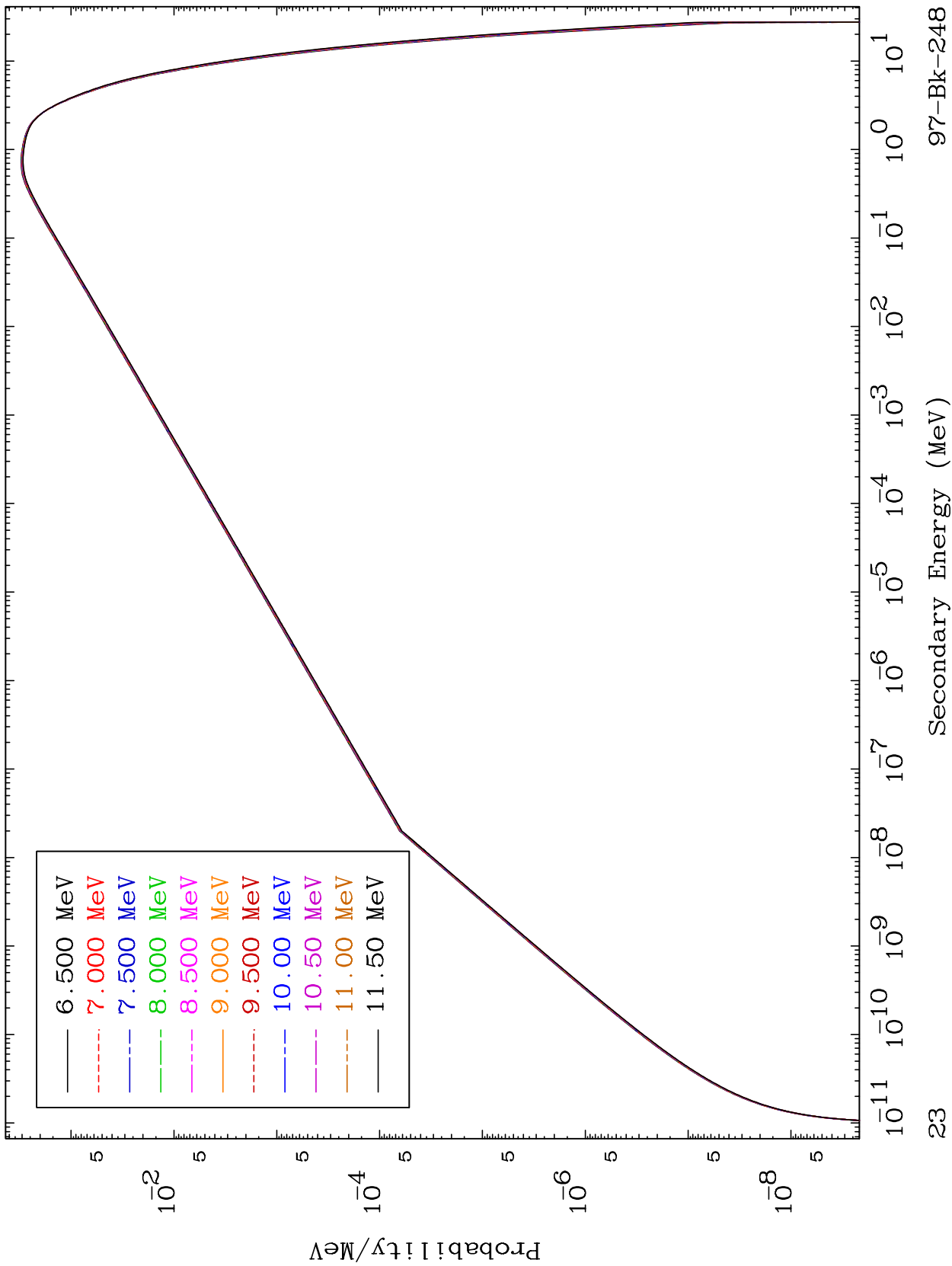
Secondary Energy (MeV)

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### Fission Energy Distribution

97-Bk-248



97-Bk-248

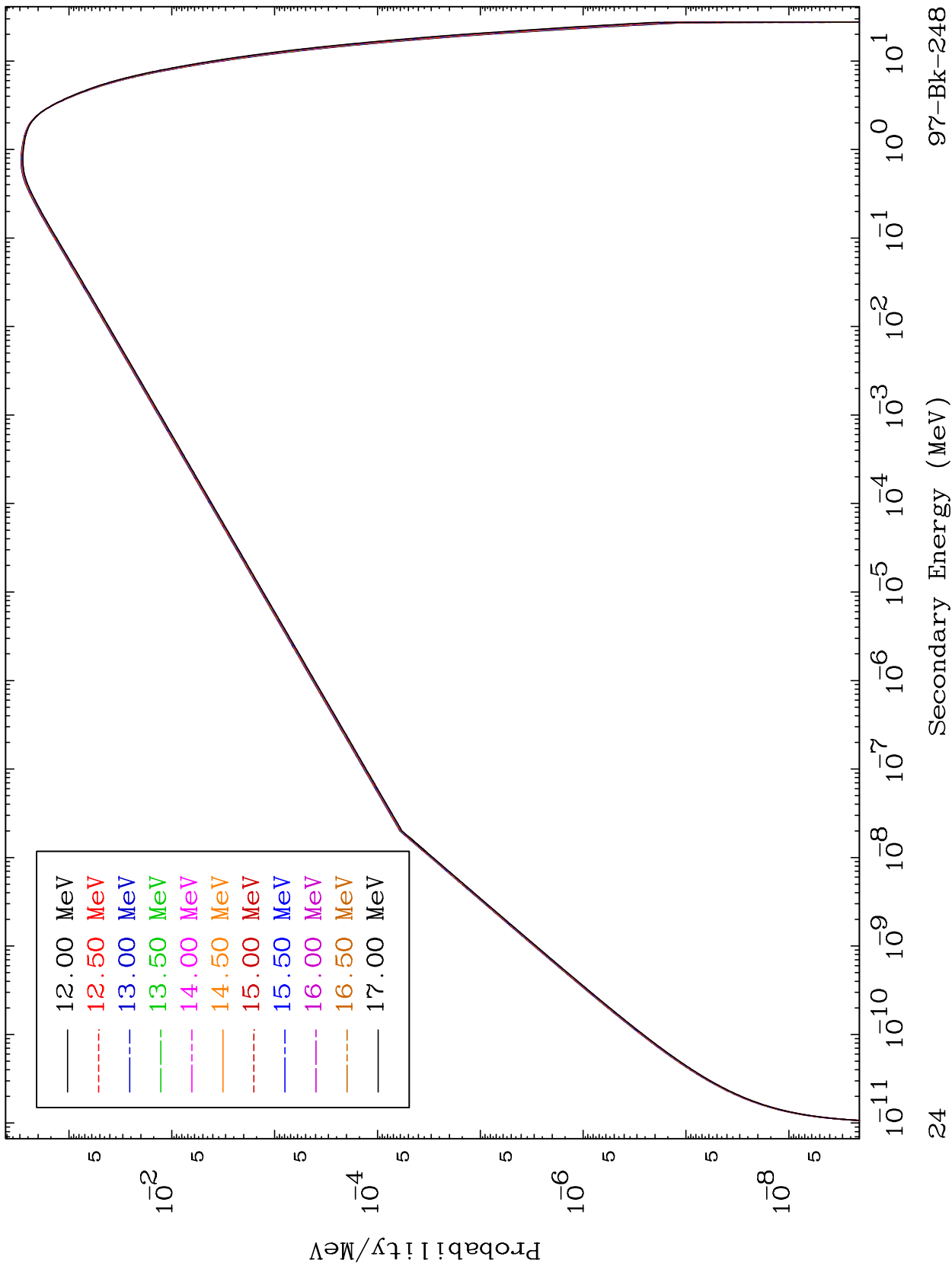
Secondary Energy (MeV)



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### Fission Energy Distribution

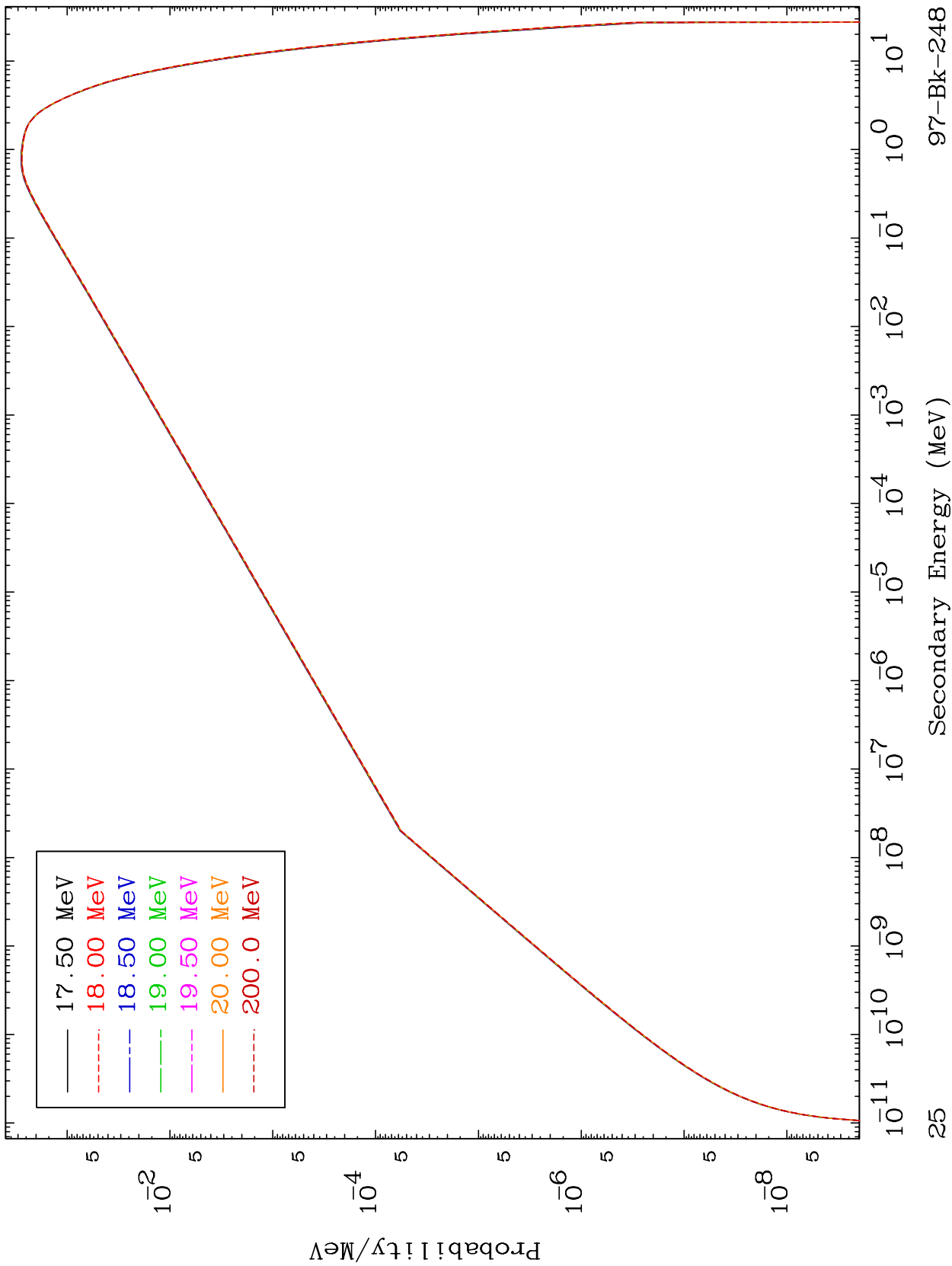
97-Bk-248



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Fission Energy Distribution

97-Bk-248

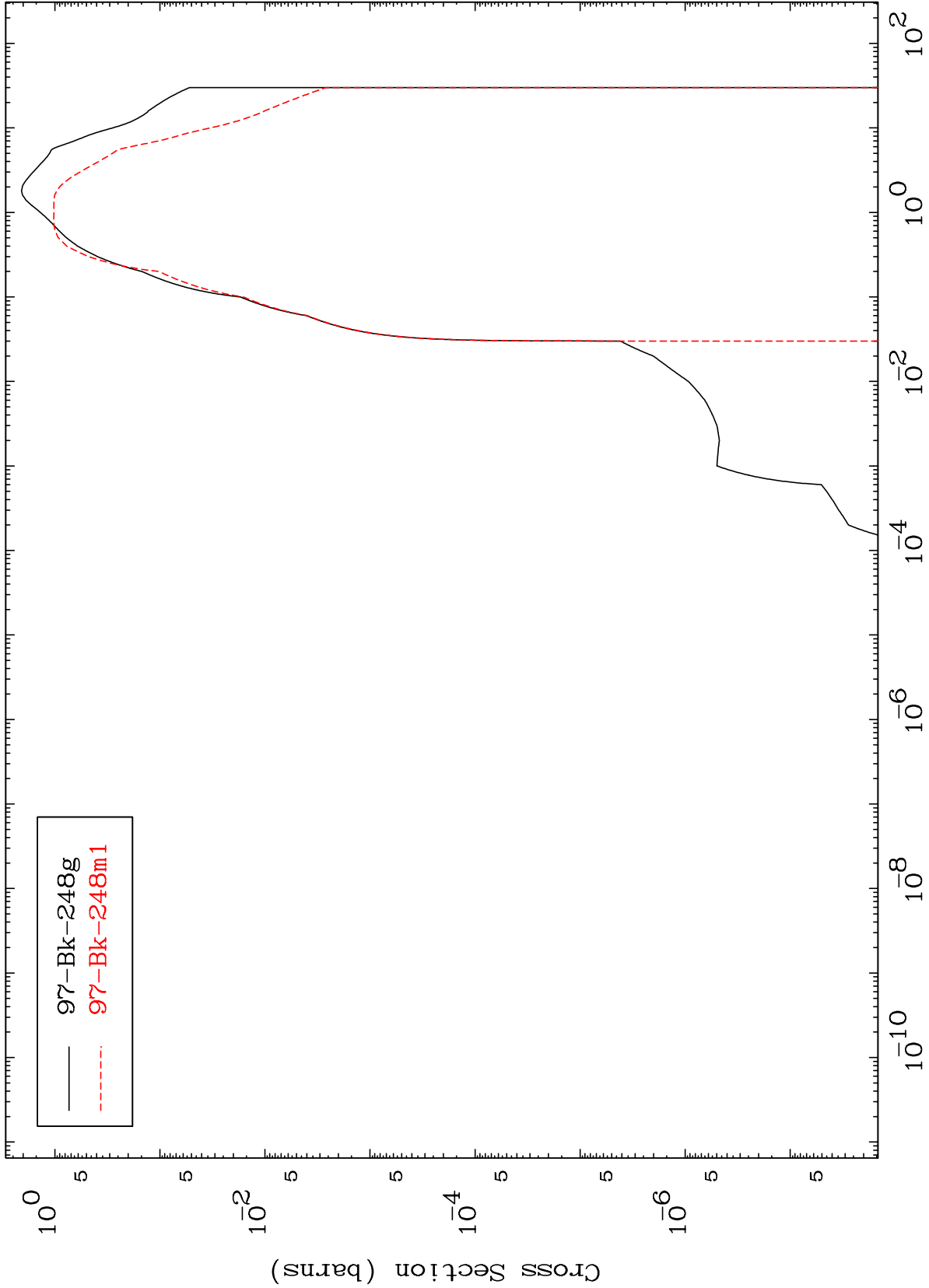


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Inelastic

97-Bk-248

Radionuclide Production Cross Section



97-Bk-248g  
97-Bk-248m1

Incident Energy (MeV)

97-Bk-248

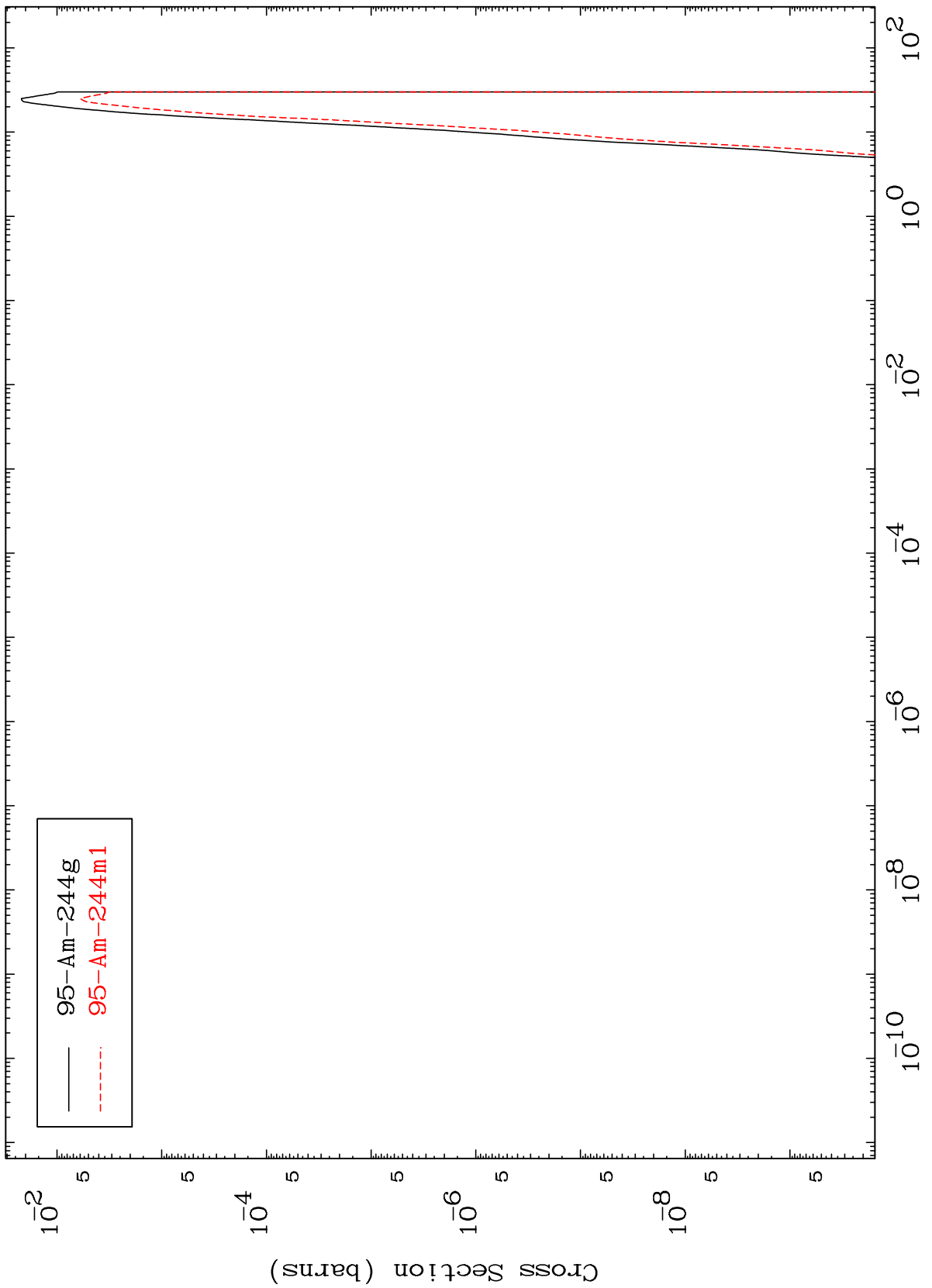
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(n,n')  $\alpha$

97-Bk-248

Radionuclide Production Cross Section



95-Am-244g  
95-Am-244m1

Incident Energy (MeV)

97-Bk-248

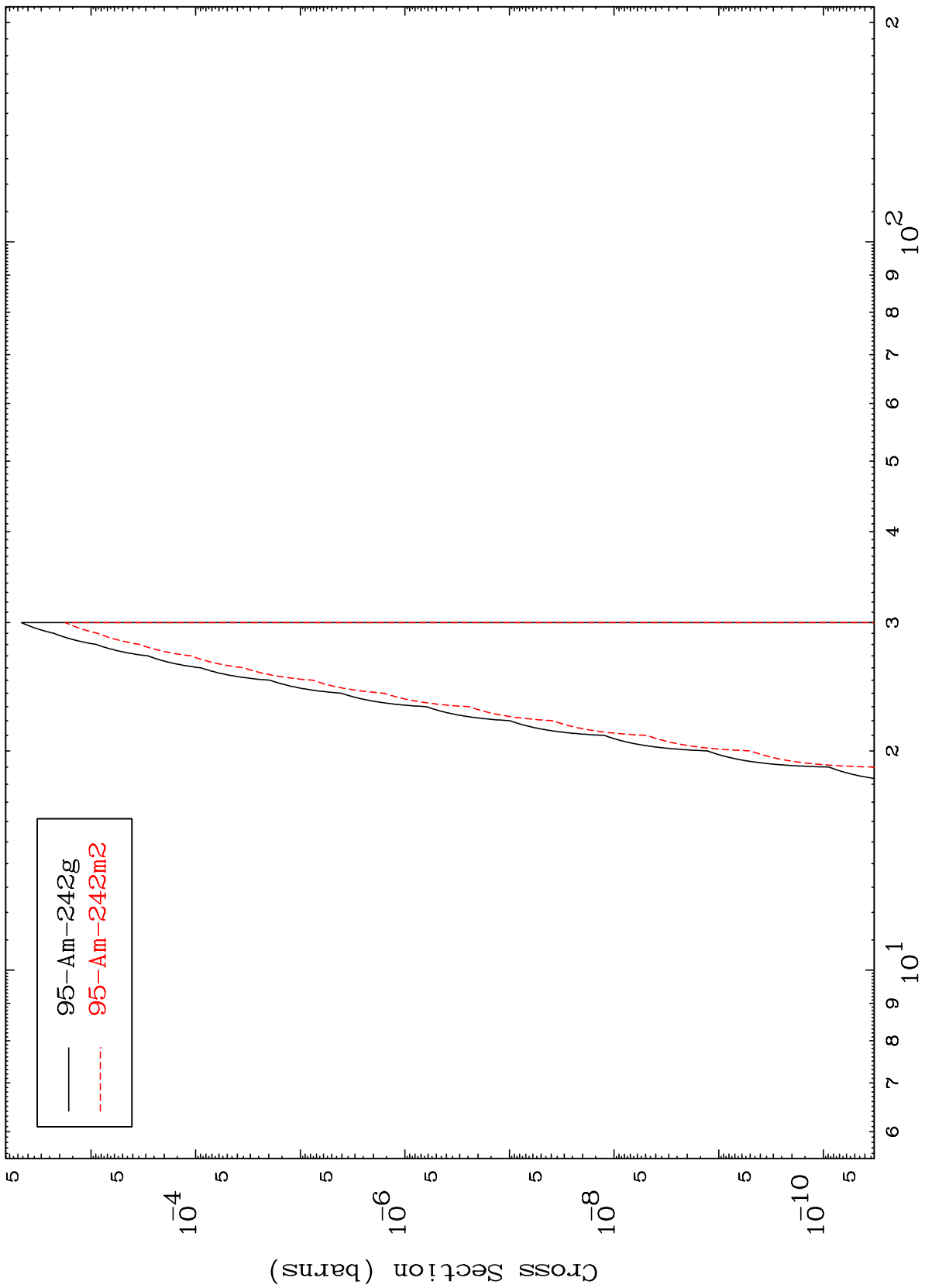
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(n,3n)  $\alpha$

97-Bk-248

Radionuclide Production Cross Section



95-Am-242g  
95-Am-242m2

28

Incident Energy (MeV)

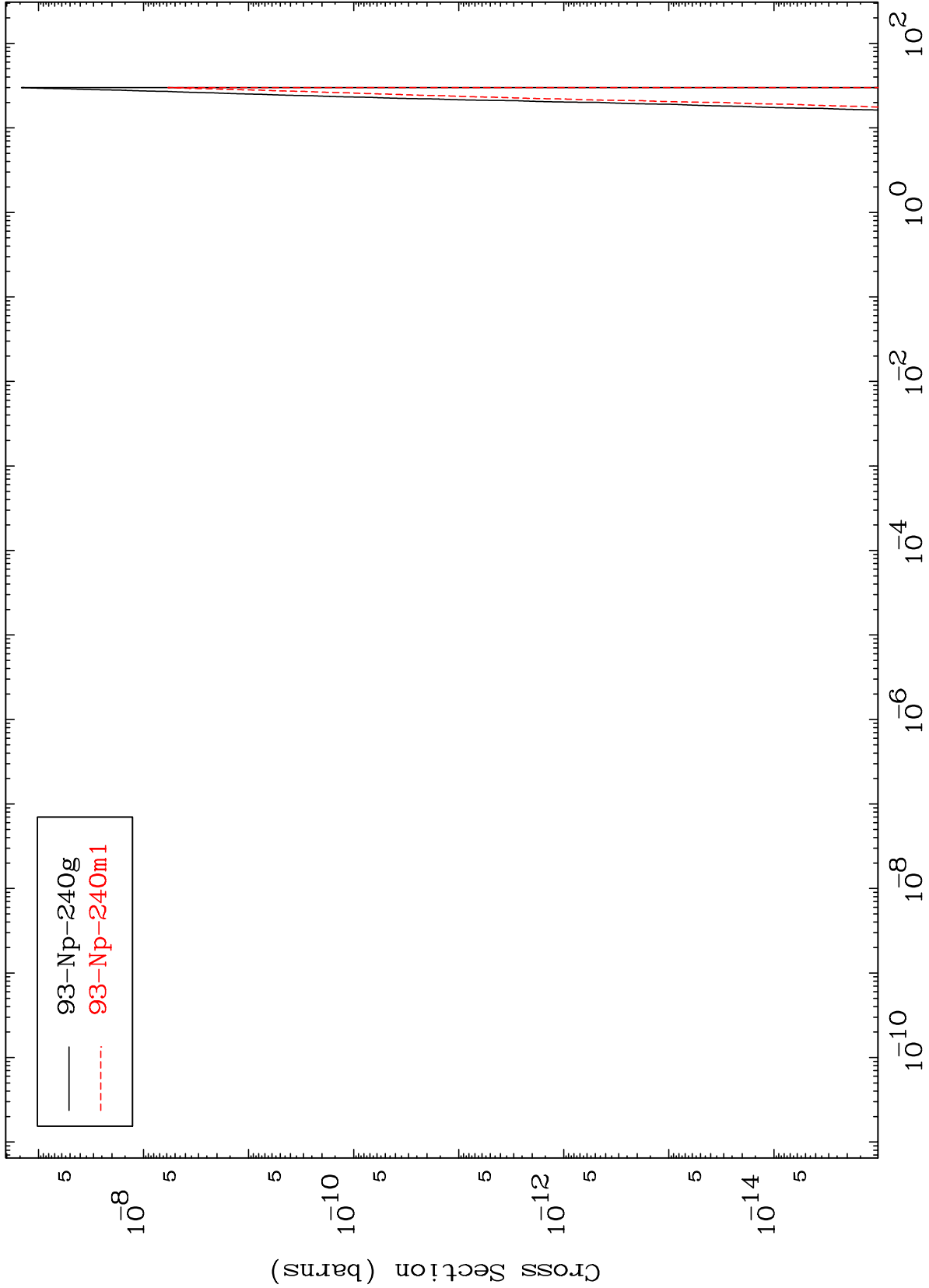
97-Bk-248

MAT 9750

(n,n') 2 $\alpha$

97-Bk-248

Radionuclide Production Cross Section

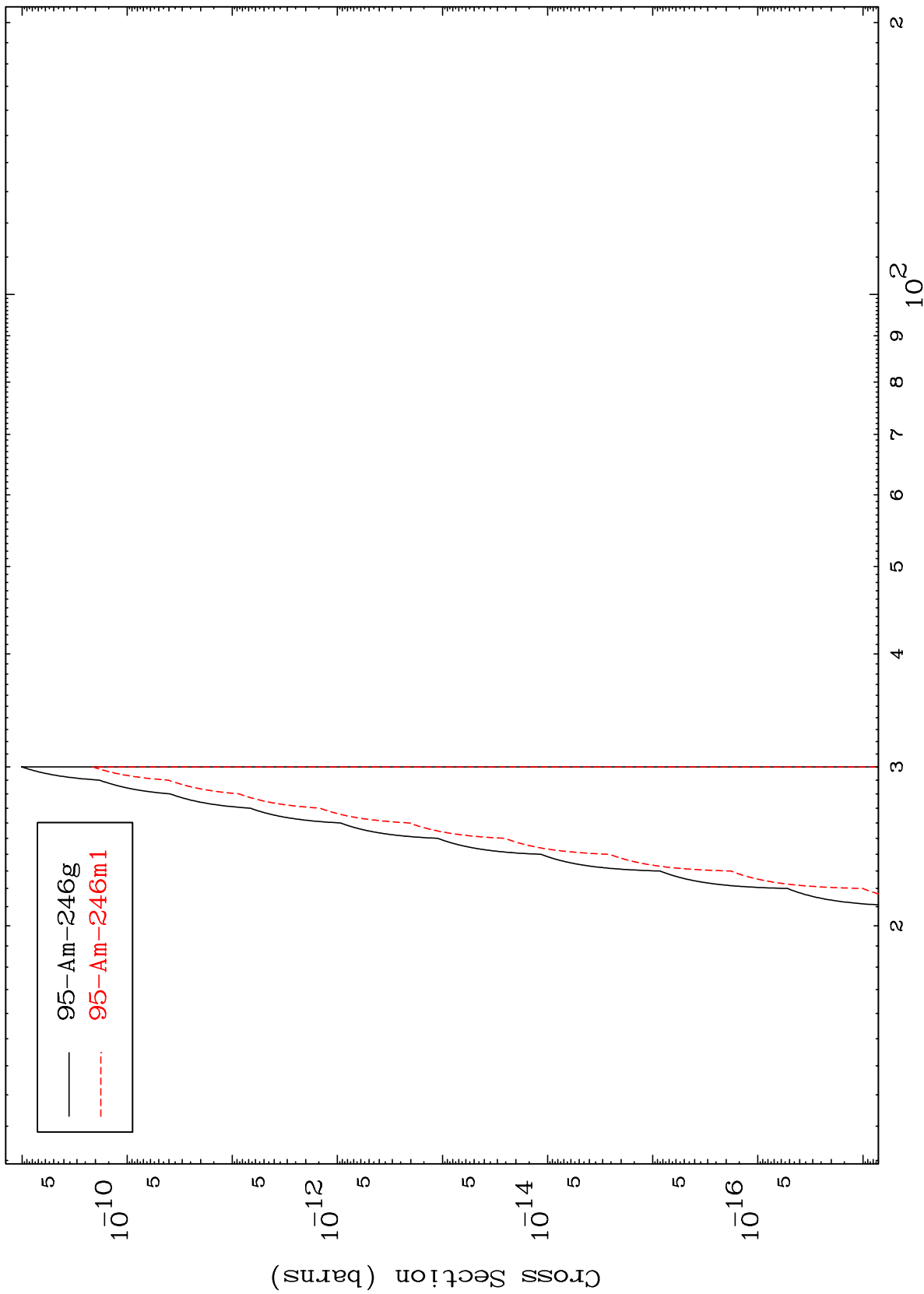


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(n,2n) p

97-Bk-248

Radionuclide Production Cross Section



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Incident Energy (MeV)

97-Bk-248

MAT 9750

97-Bk-248

Radionuclide Production Cross Section  
(n,He-3)

