

Program EVALPLOT  
(Version 2018-1)

by

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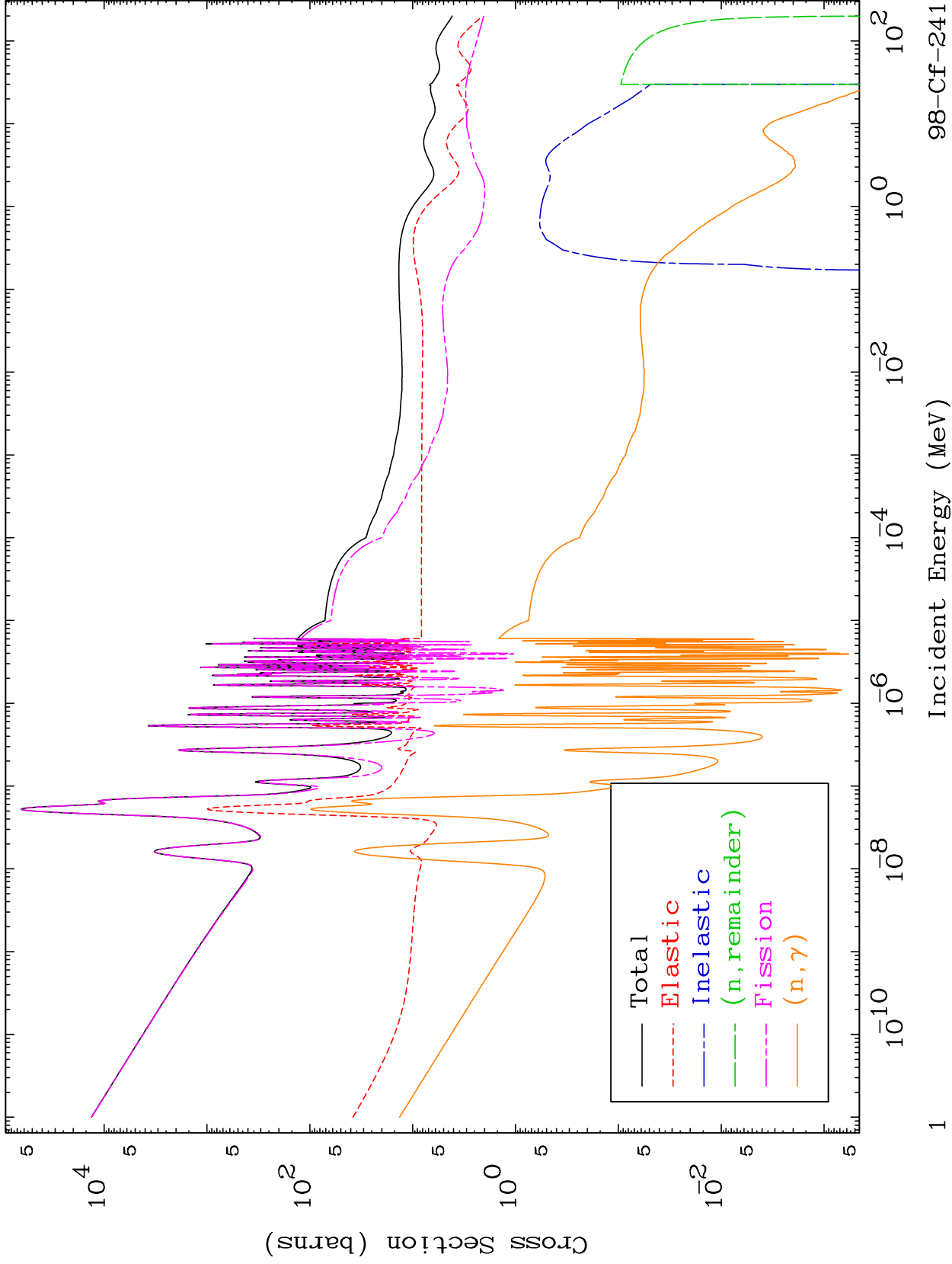
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Press Mouse Button to Start

MAT 9828

Major  
293 Kelvin Cross Sections

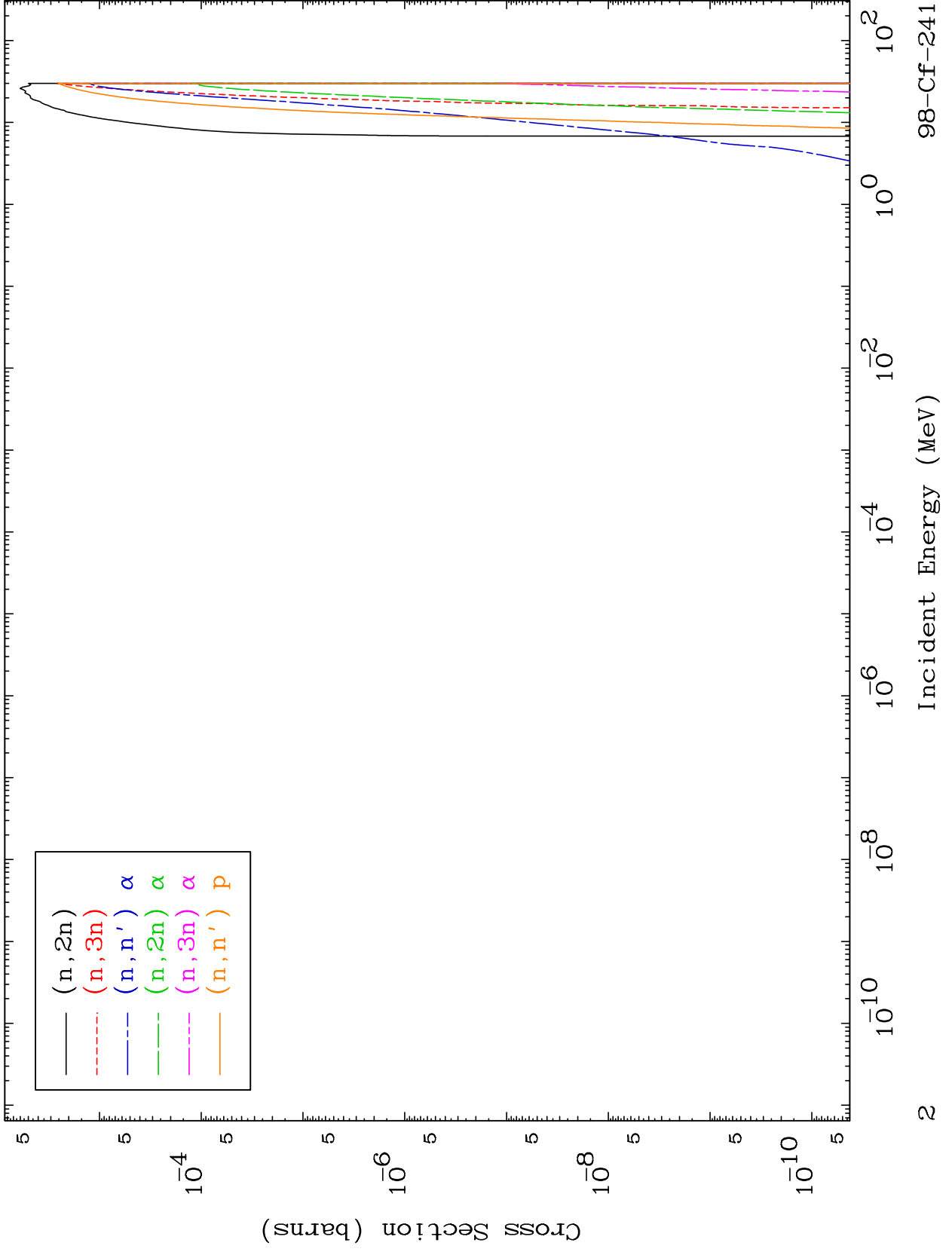
98-Cf-241



MAT 9828

Neutron Production  
293 Kelvin Cross Sections

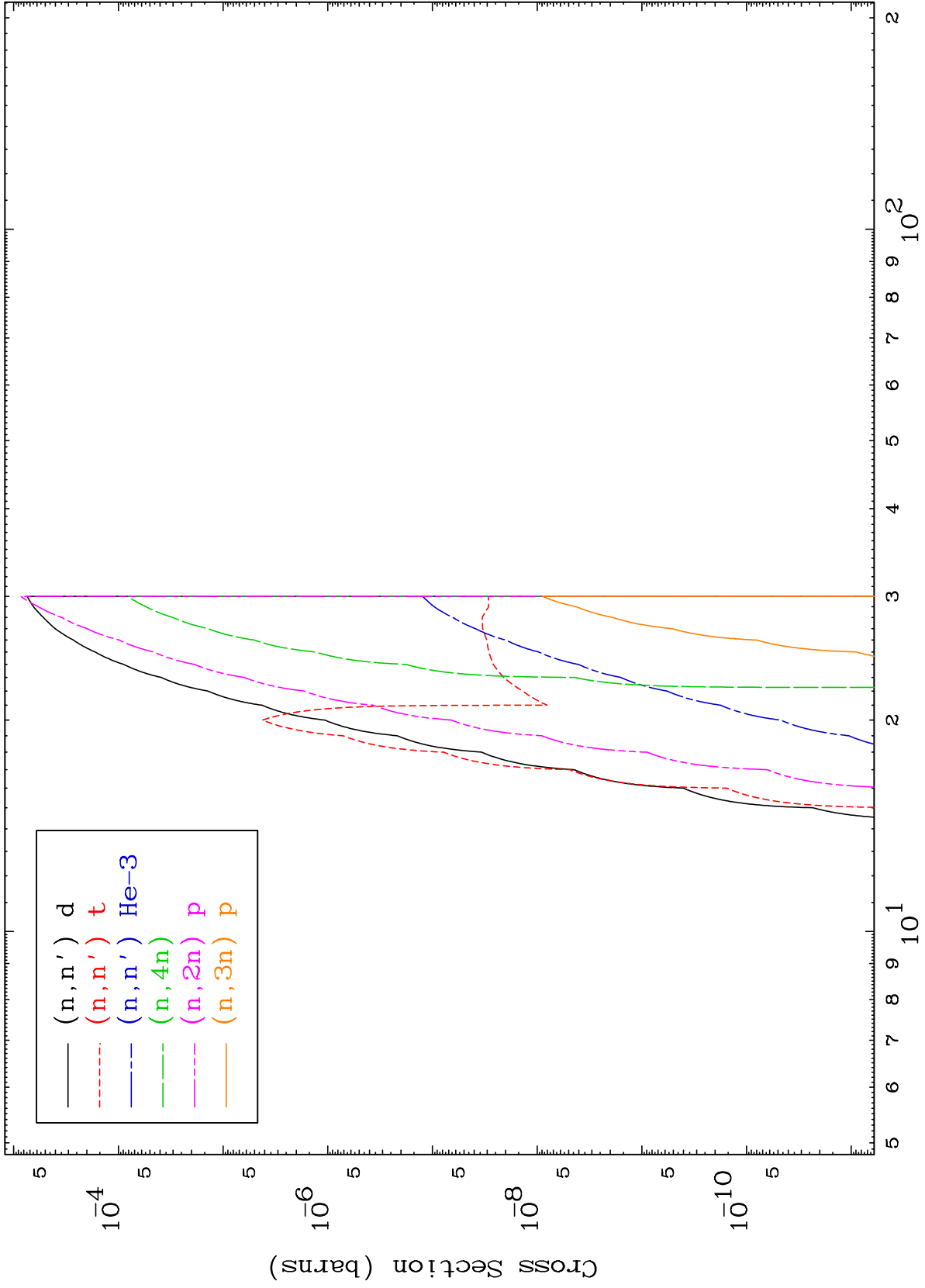
98-Cf-241



MAT 9828

Neutron Production  
293 Kelvin Cross Sections

98-Cf-241



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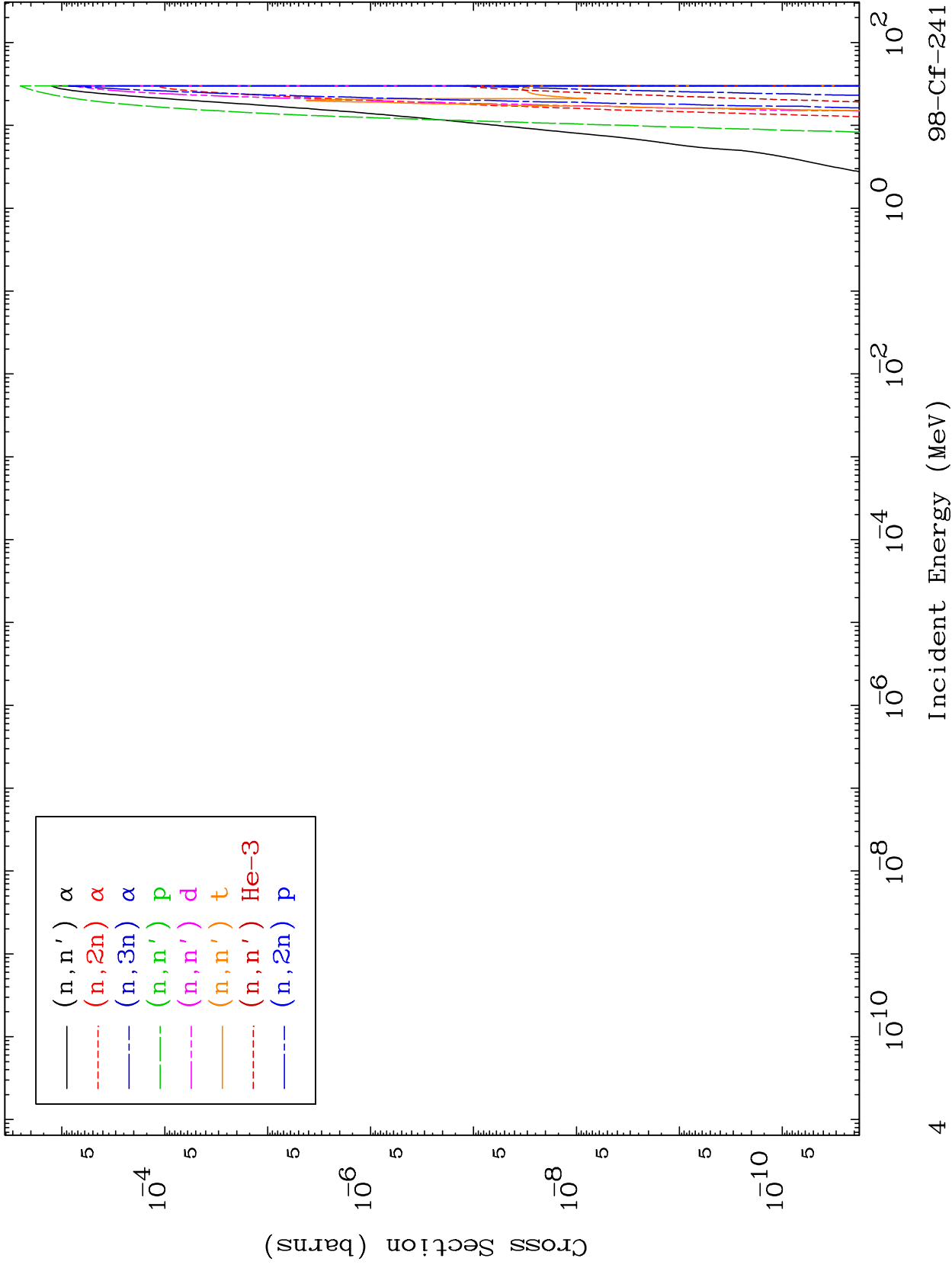
Incident Energy (MeV)

98-Cf-241

MAT 9828

Charged Particle  
293 Kelvin Cross Sections

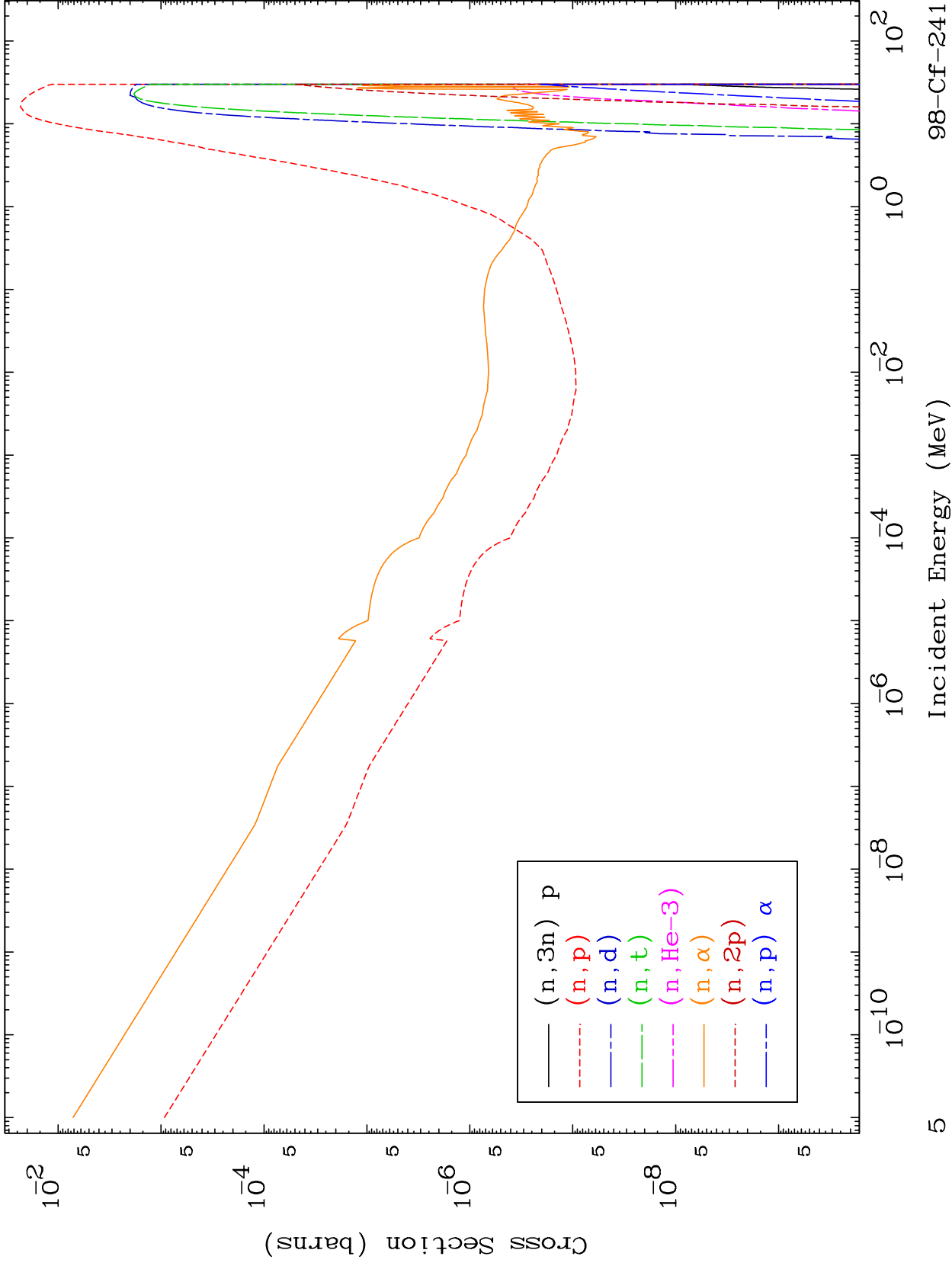
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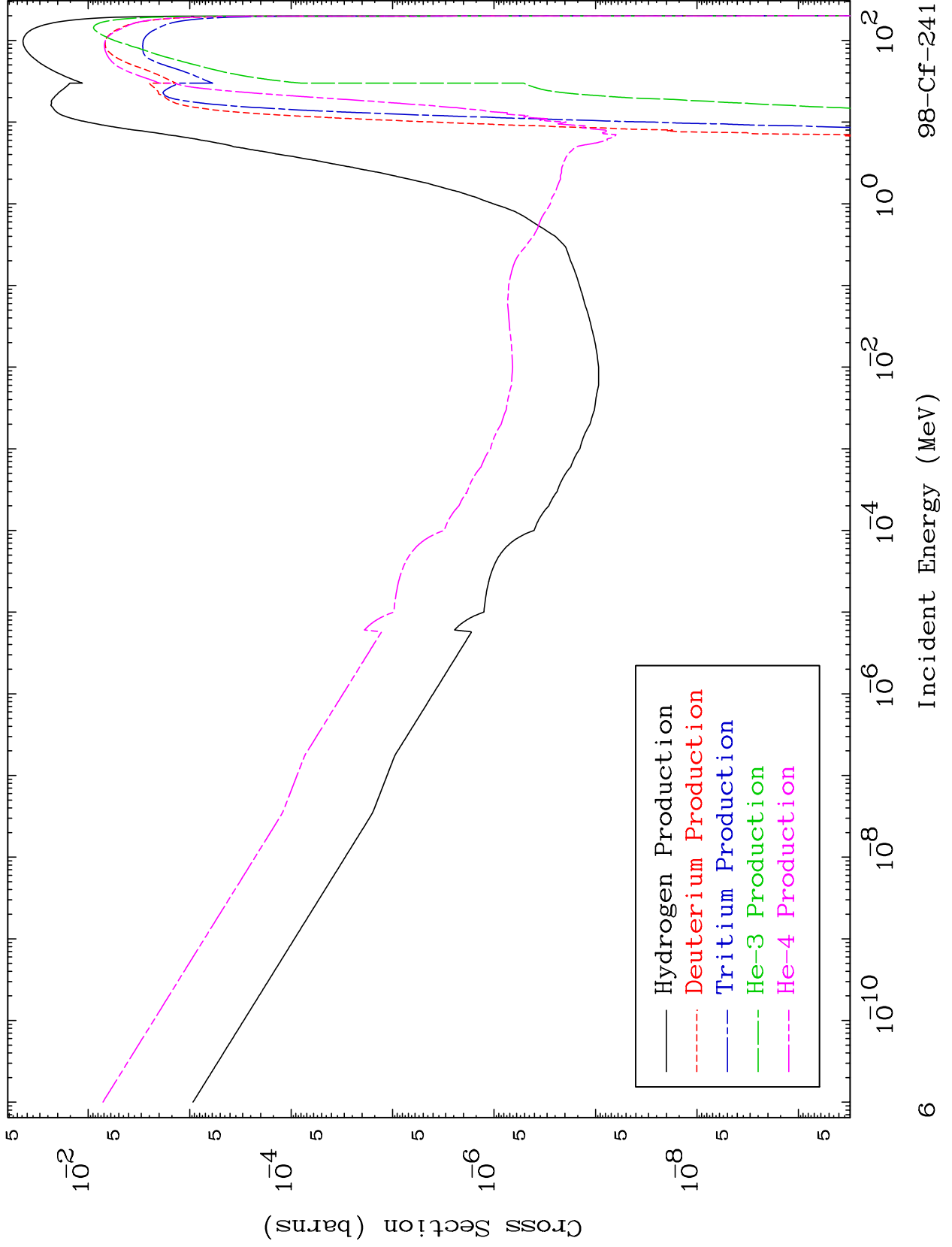


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Charged Particle  
293 Kelvin Cross Sections

98-Cf-241

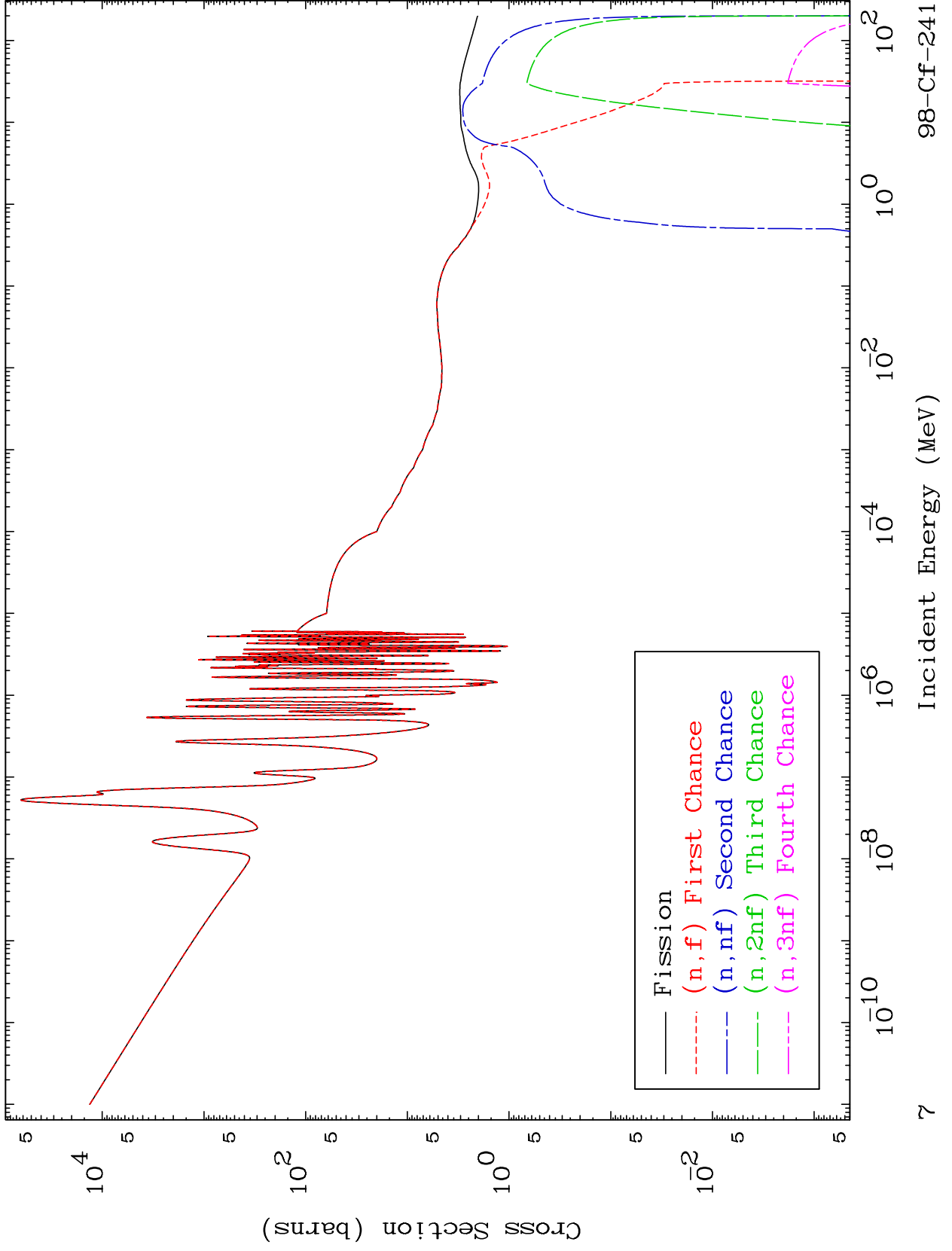




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Fission  
293 Kelvin Cross Sections

98-Cf-241



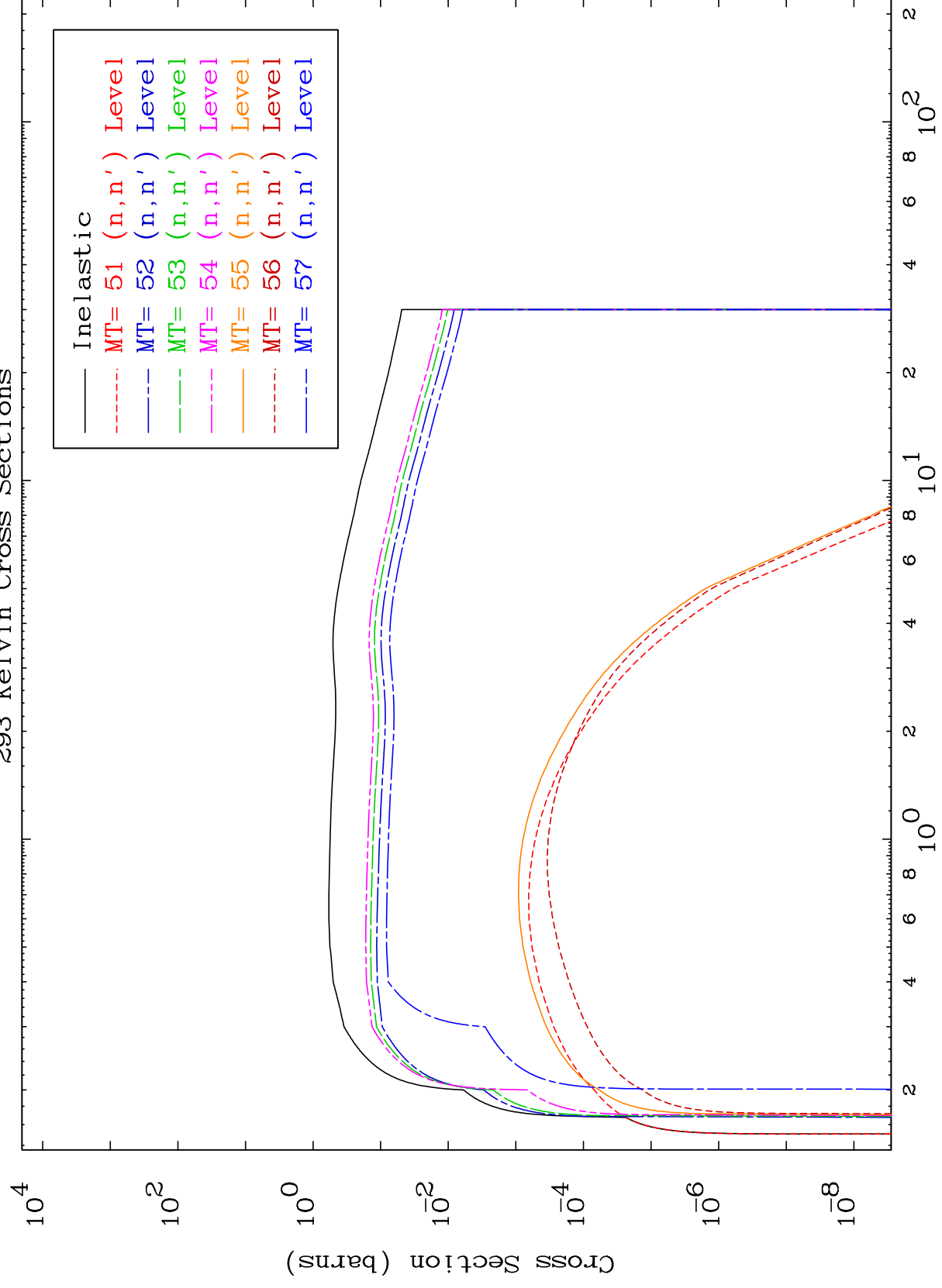


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(n,n') Level

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293 Kelvin Cross Sections



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Incident Energy (MeV)

98-Cf-241

MAT 9828

(n,n') Level

98-Cf-241

293 Kelvin Cross Sections

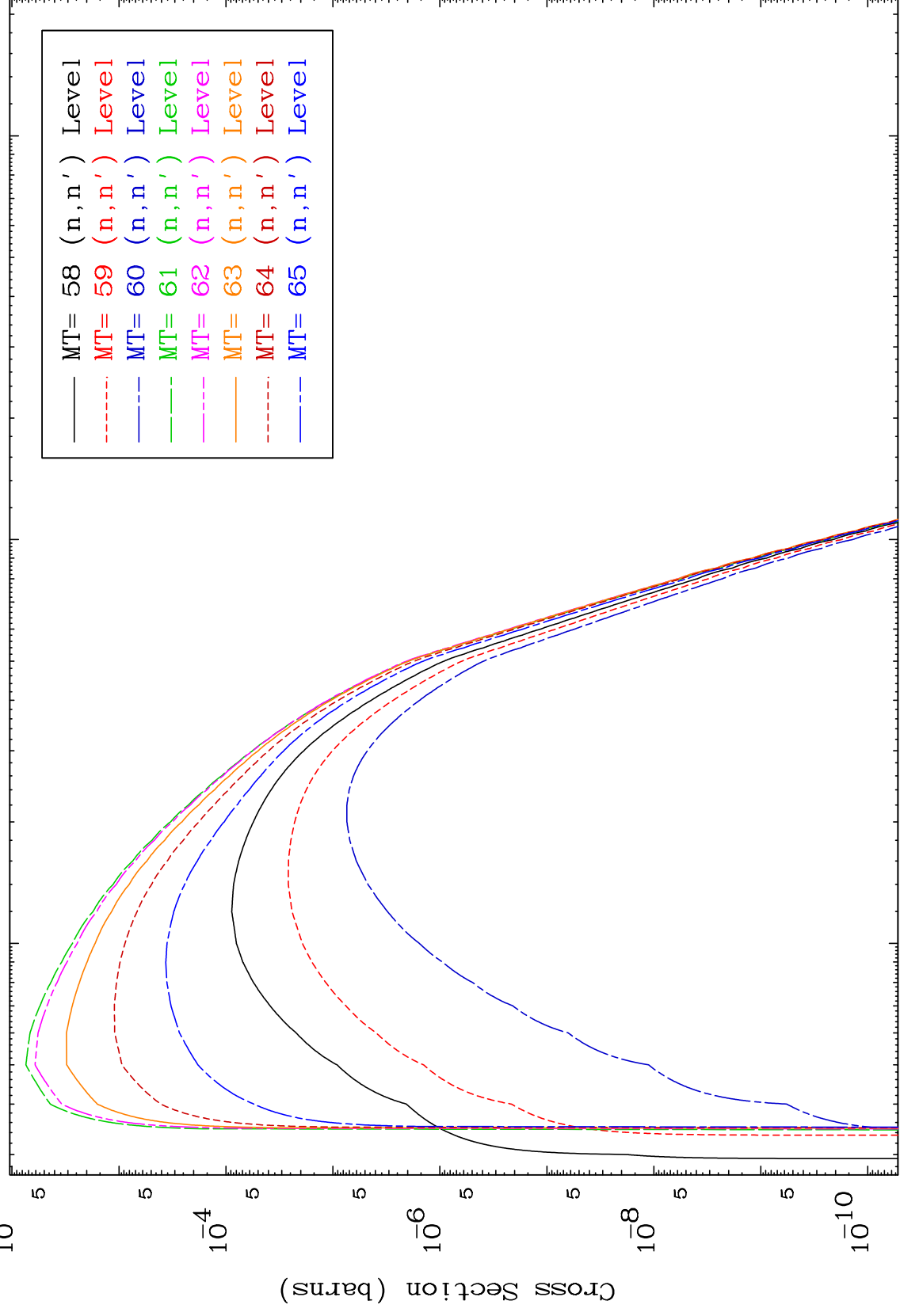
$10^{-2}$

$10^{-4}$

$10^{-6}$

$10^{-8}$

$10^{-10}$



98-Cf-241

Incident Energy (MeV)

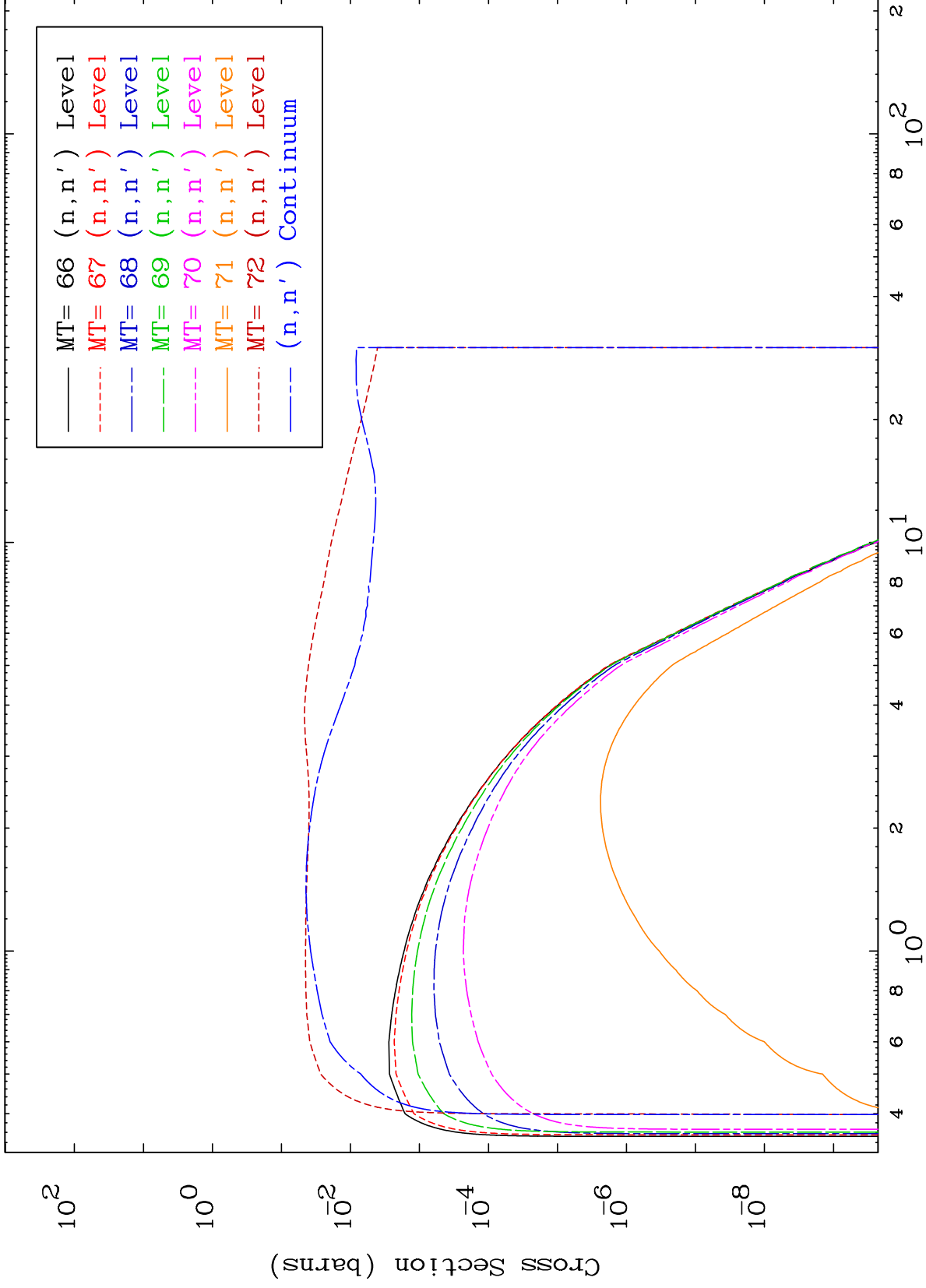
9

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(n,n') Level

98-Cf-241

293 Kelvin Cross Sections



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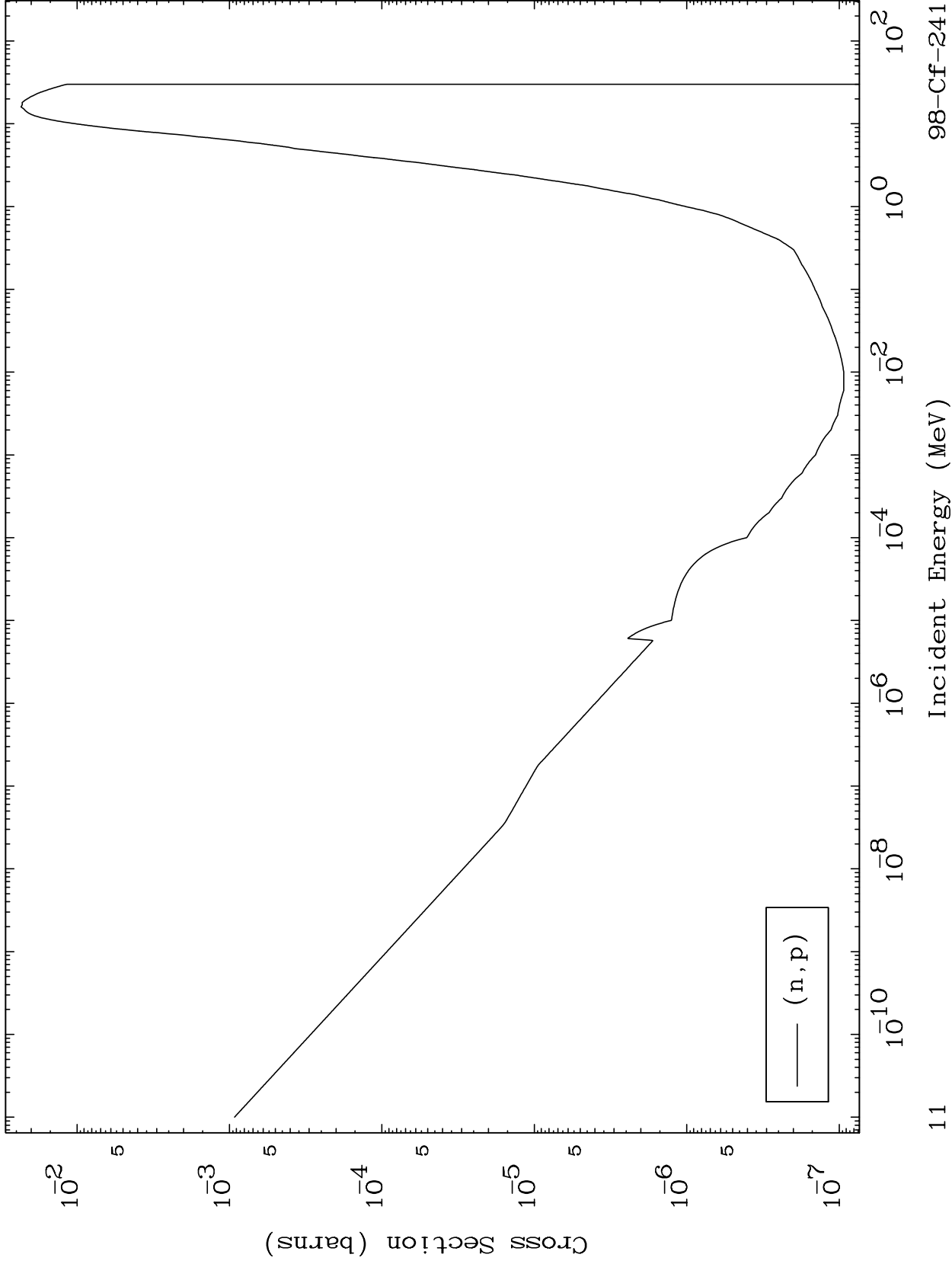
Incident Energy (MeV)

98-Cf-241

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(n,p) Levels  
293 Kelvin Cross Sections

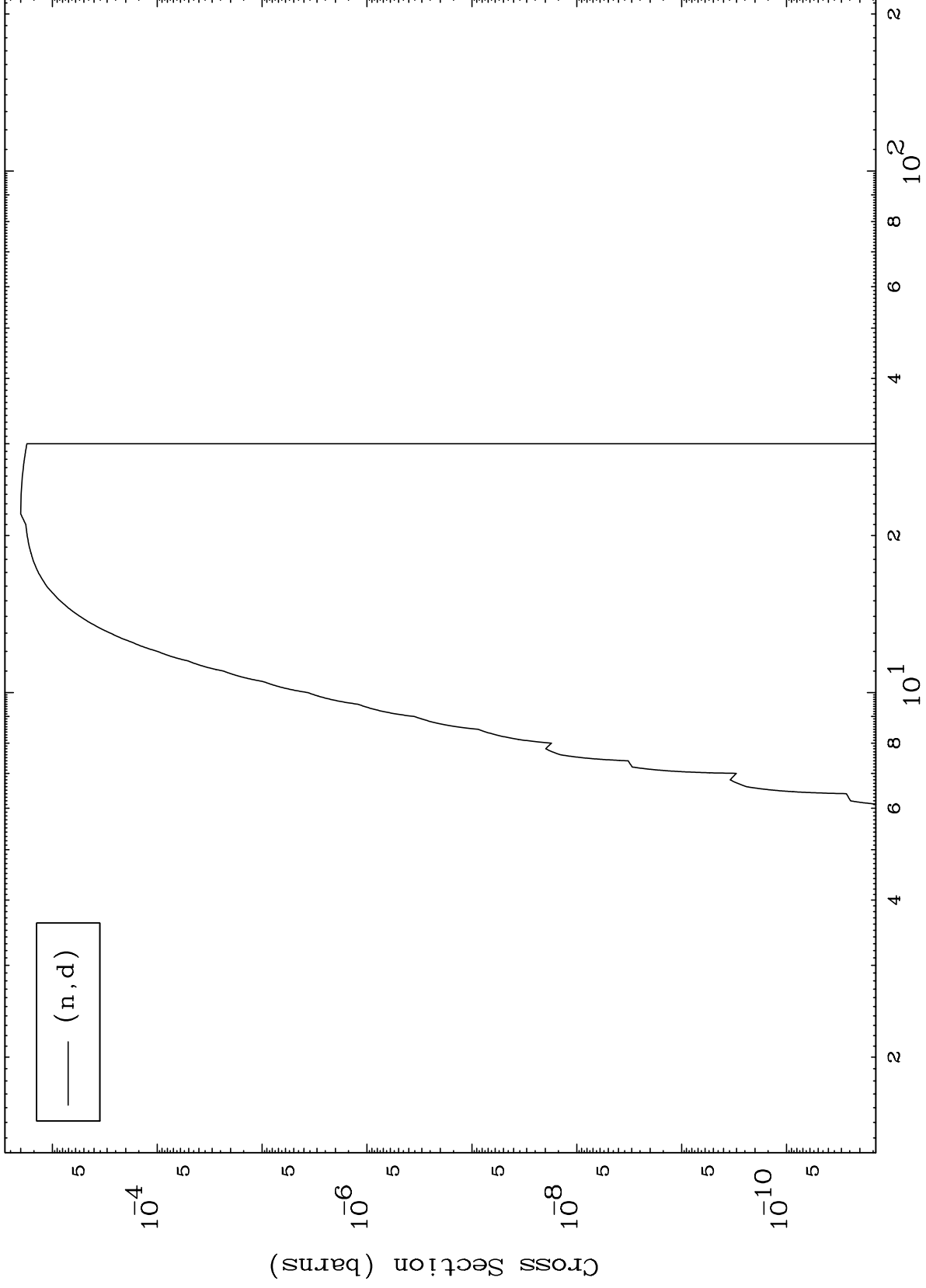
98-Cf-241



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(n,d) Levels  
293 Kelvin Cross Sections

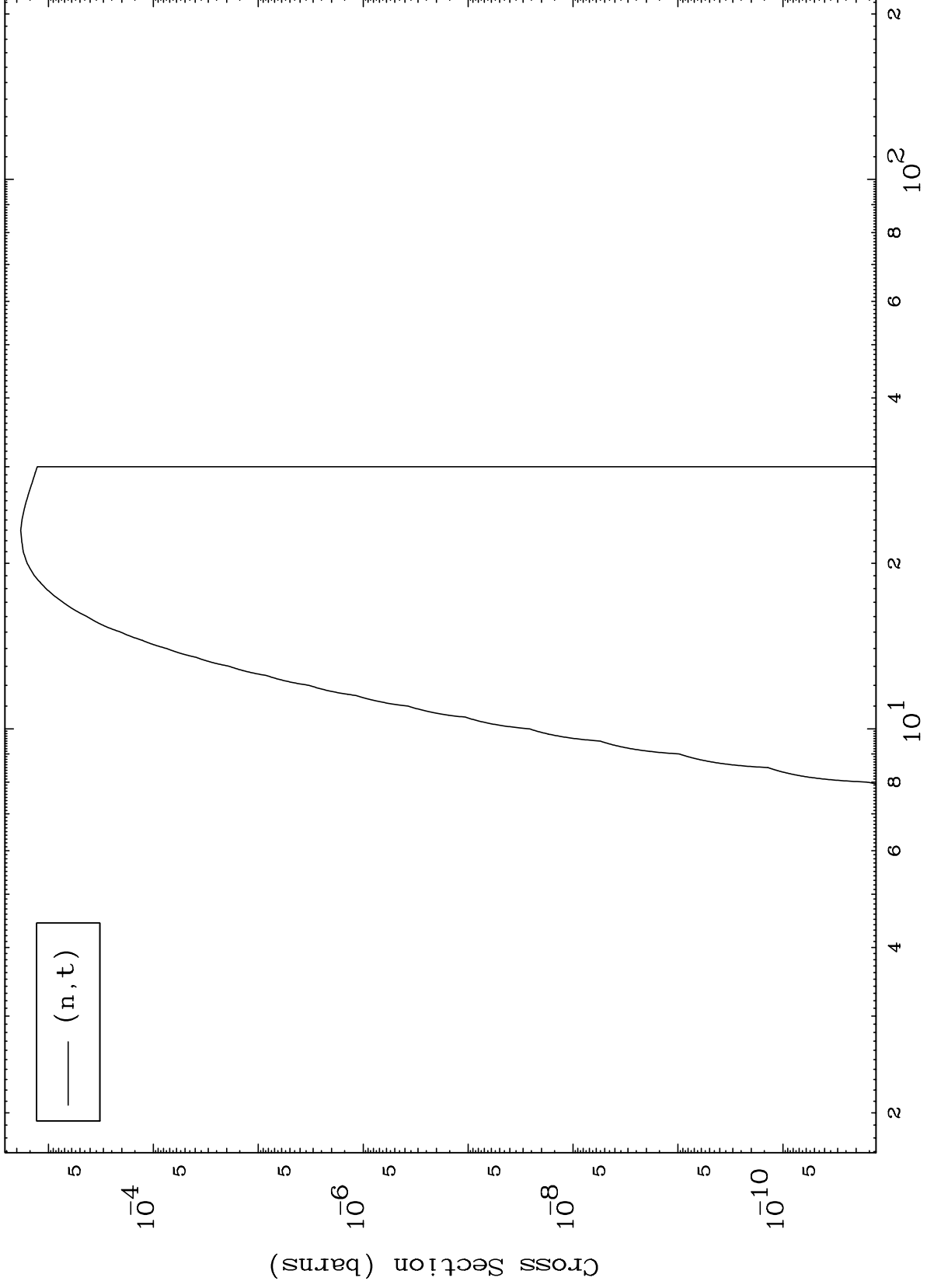
98-Cf-241



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(n,t) Levels  
293 Kelvin Cross Sections

98-Cf-241



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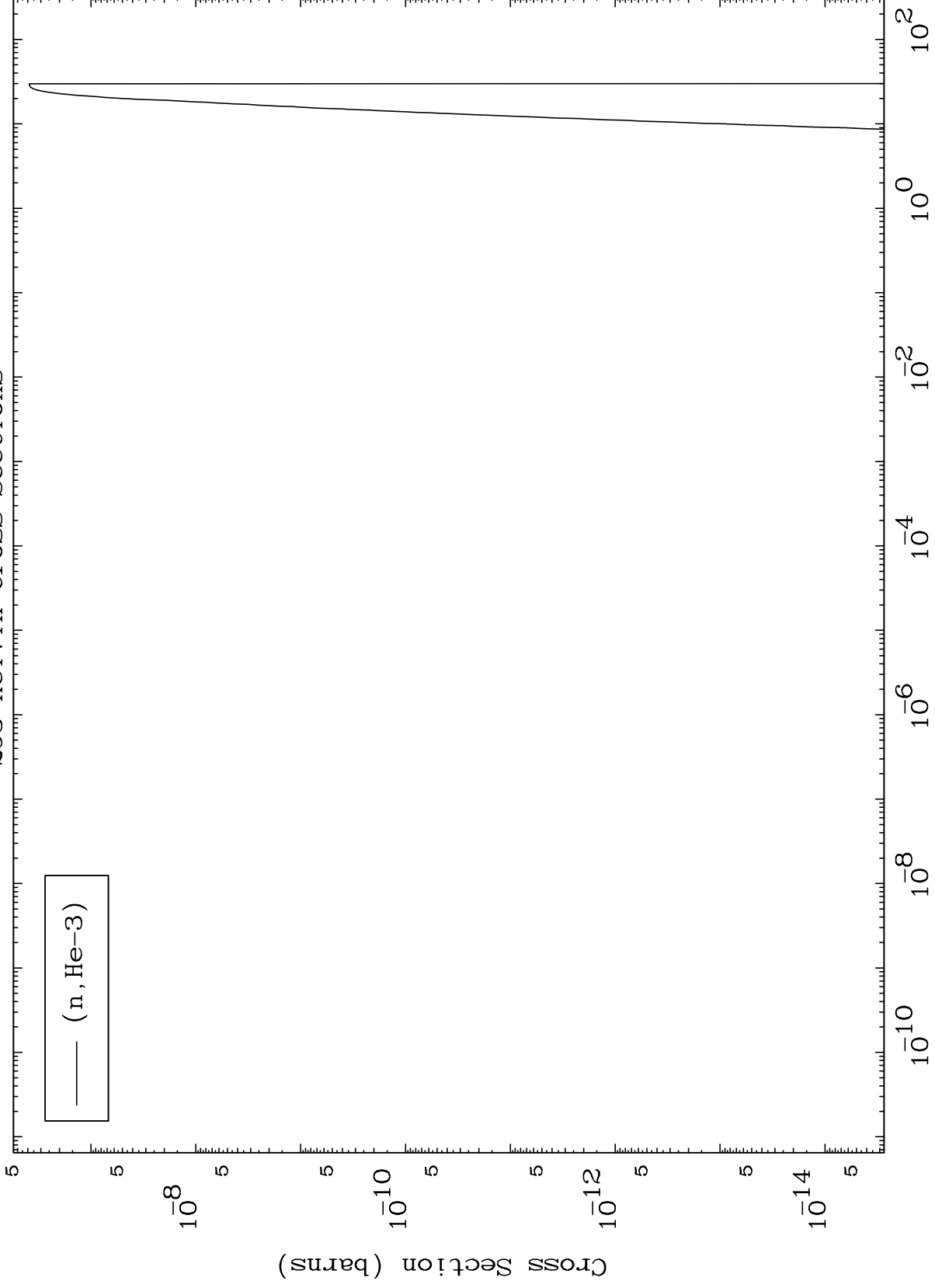
Incident Energy (MeV)

98-Cf-241

MAT 9828

(n,He3) Levels  
293 Kelvin Cross Sections

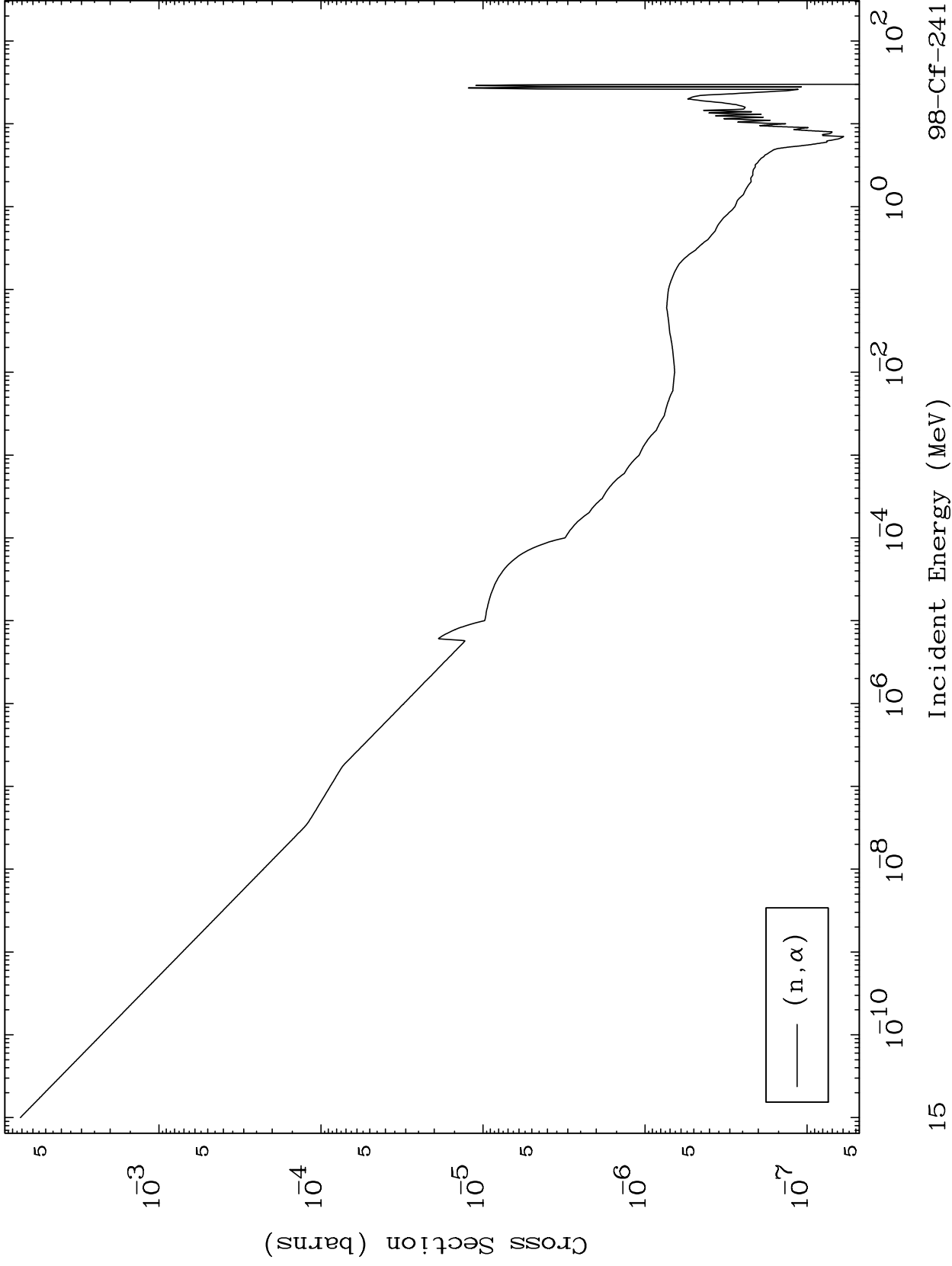
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MAT 9828

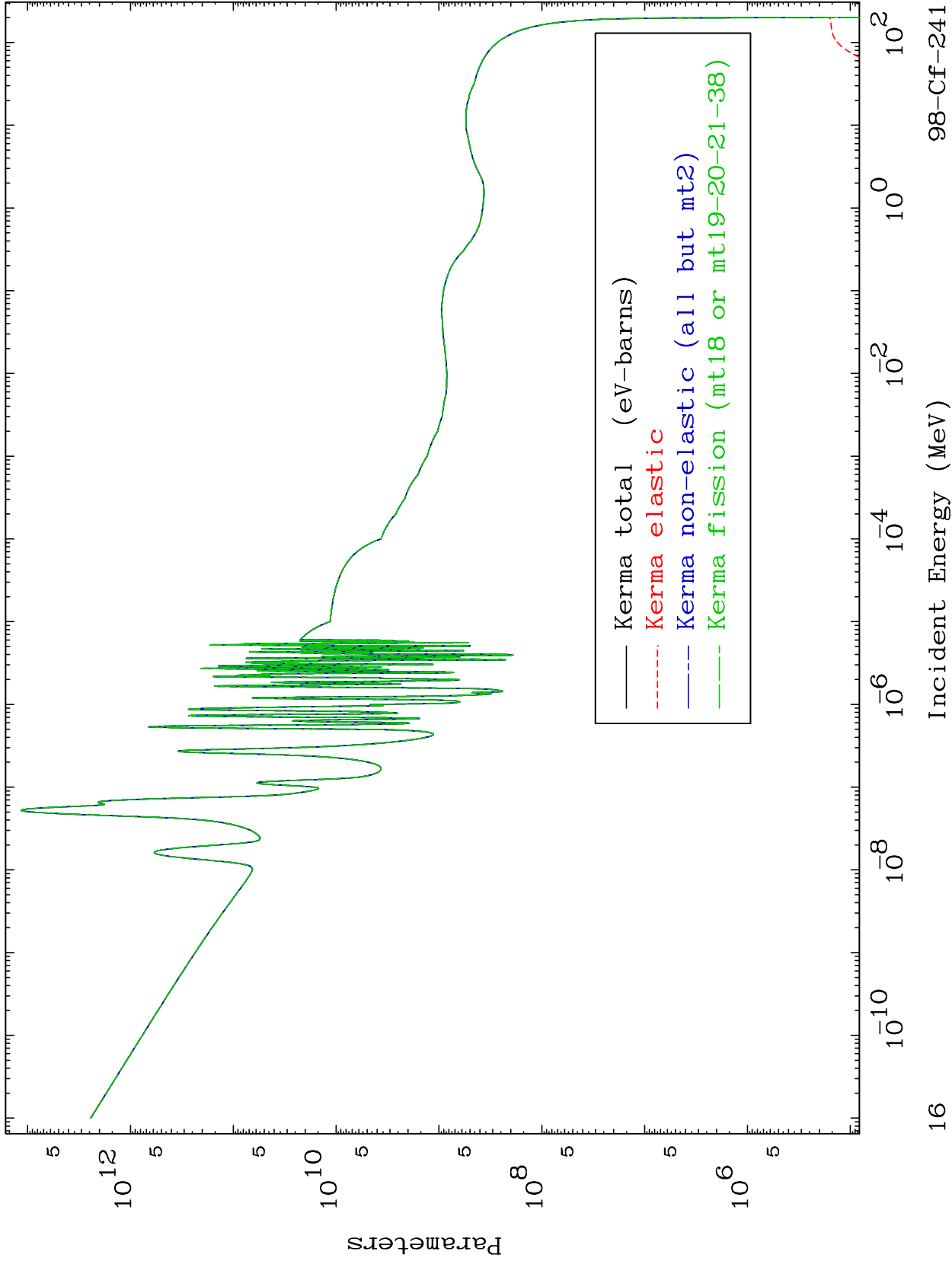
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

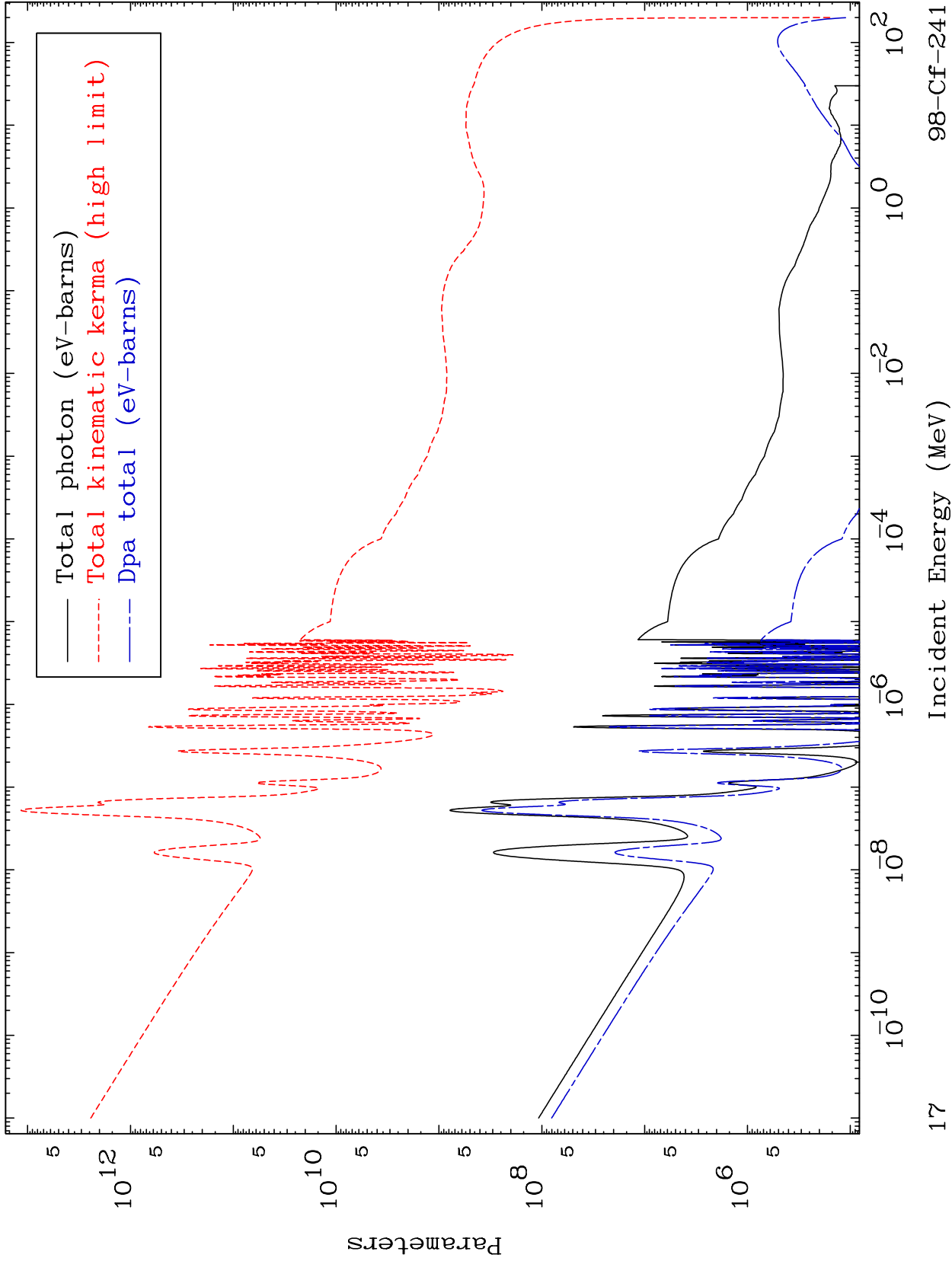
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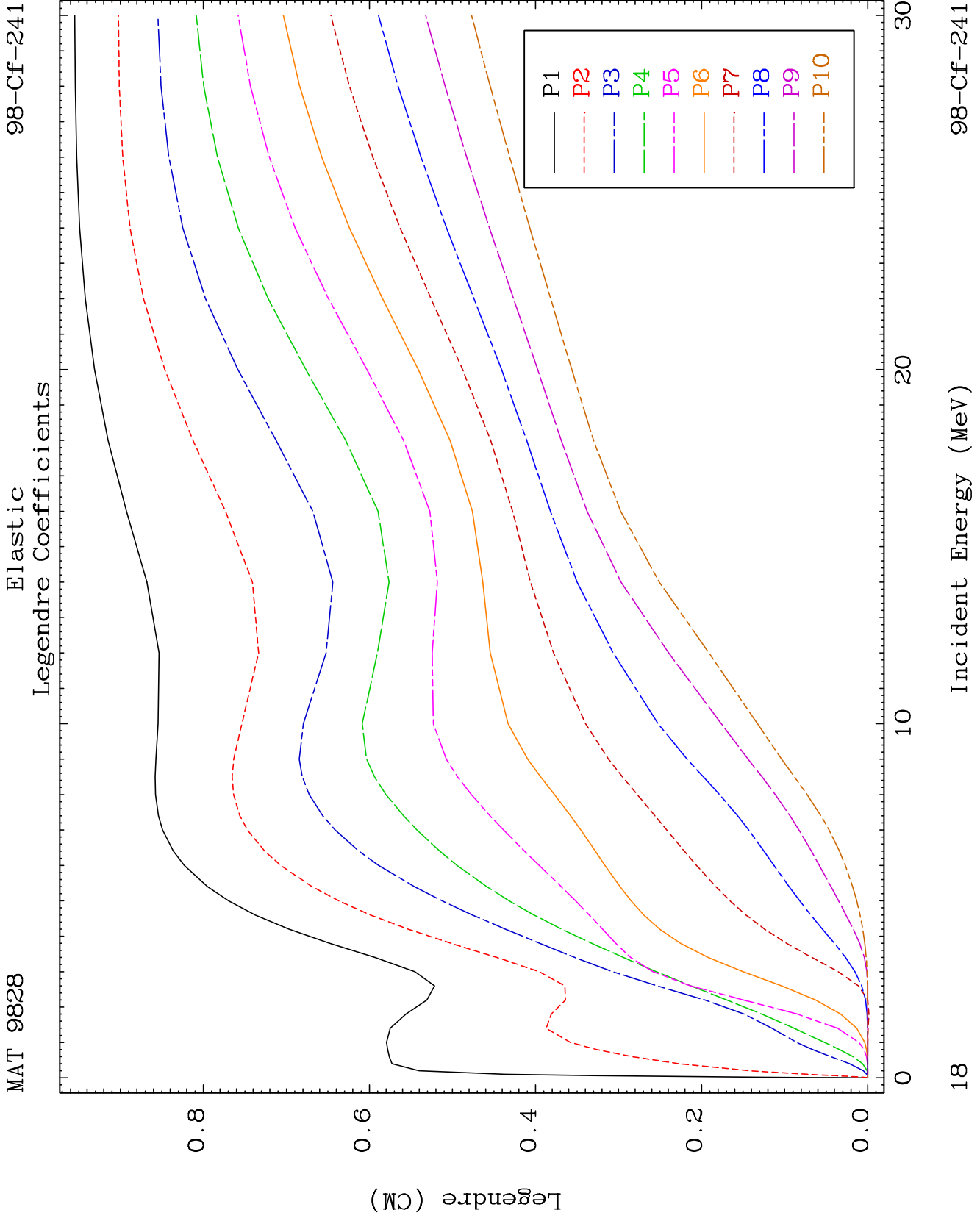


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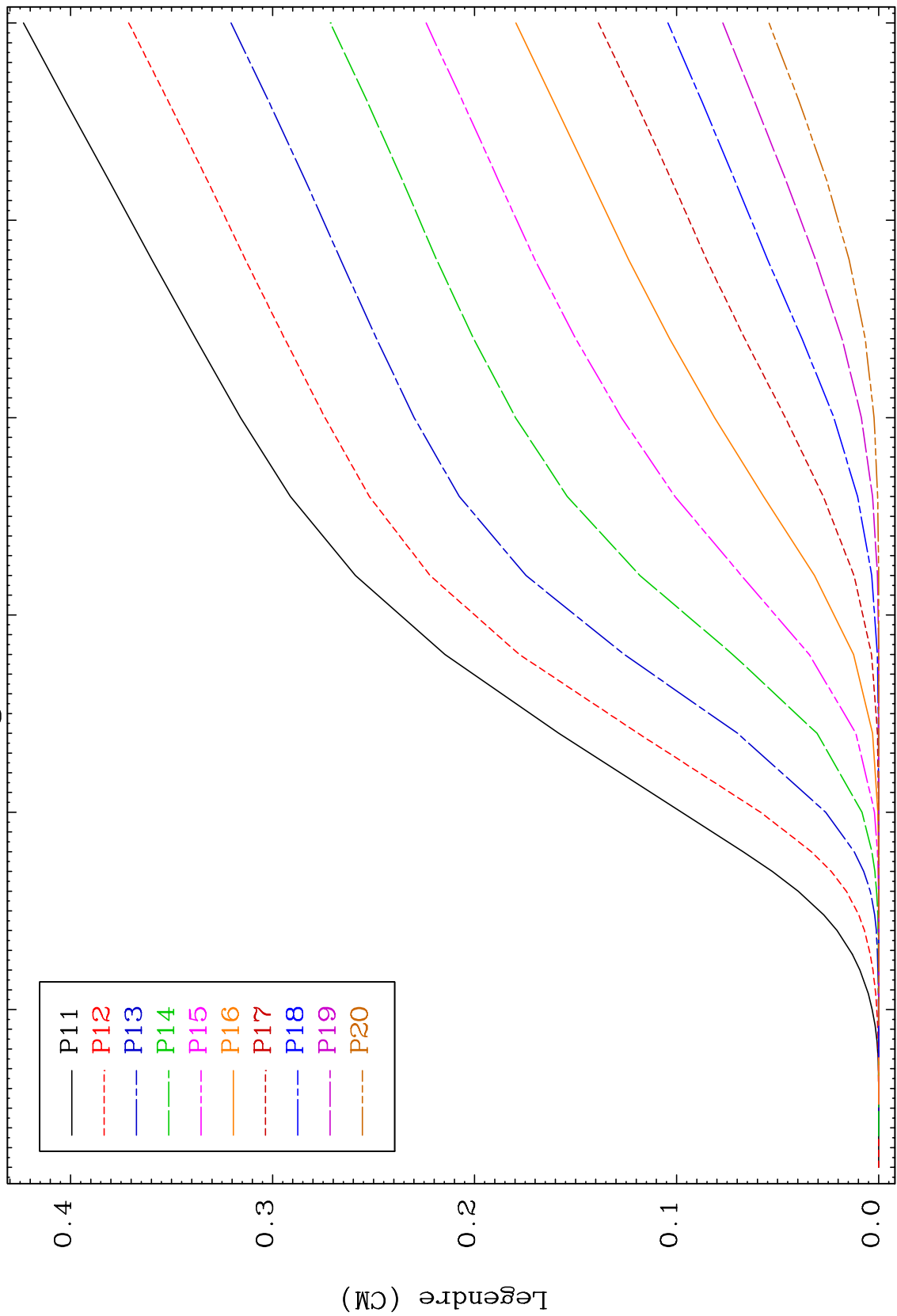




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### Elastic Legendre Coefficients

98-Cf-241



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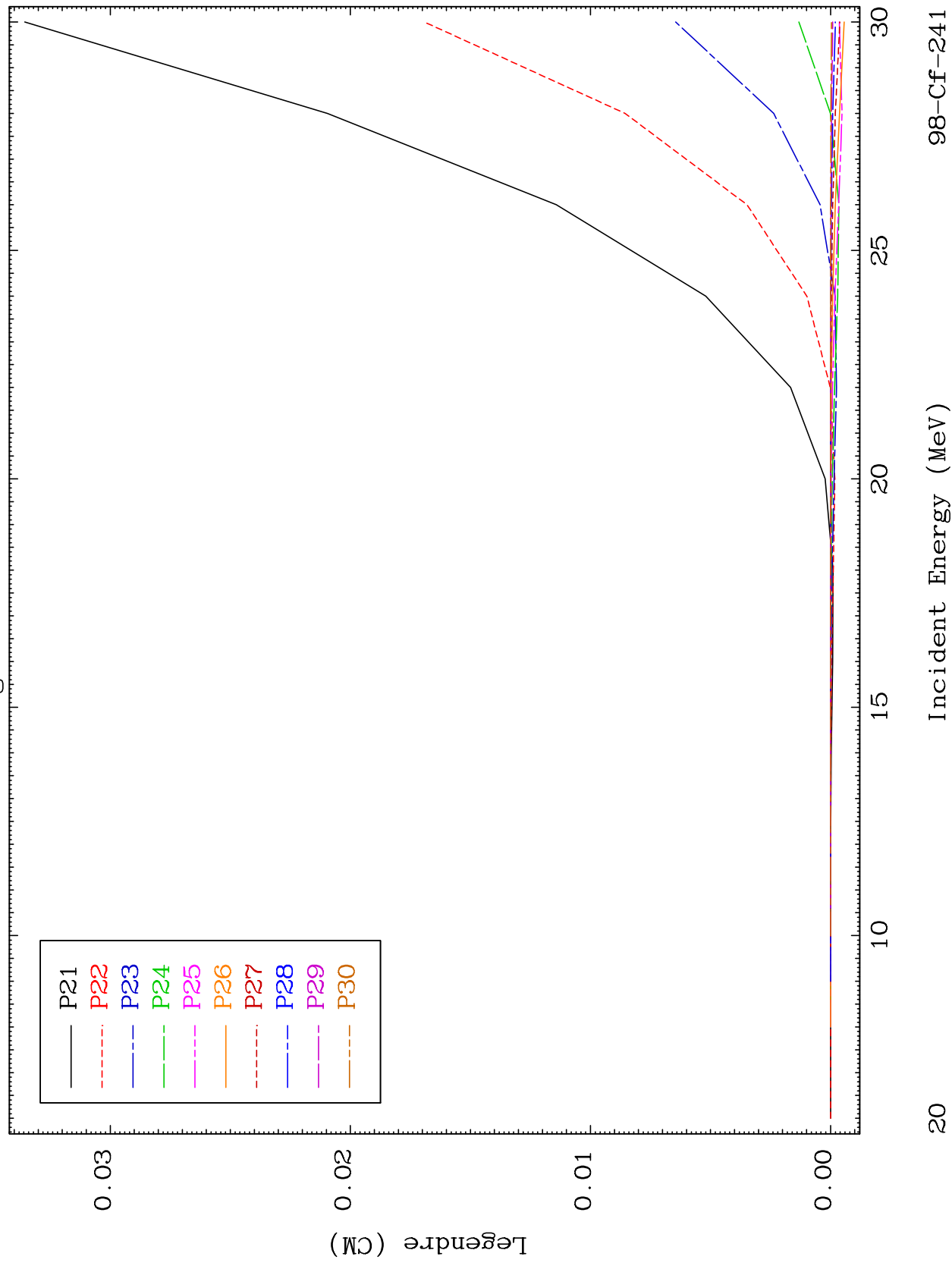
Incident Energy (MeV)

98-Cf-241

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### Elastic Legendre Coefficients

98-Cf-241



98-Cf-241

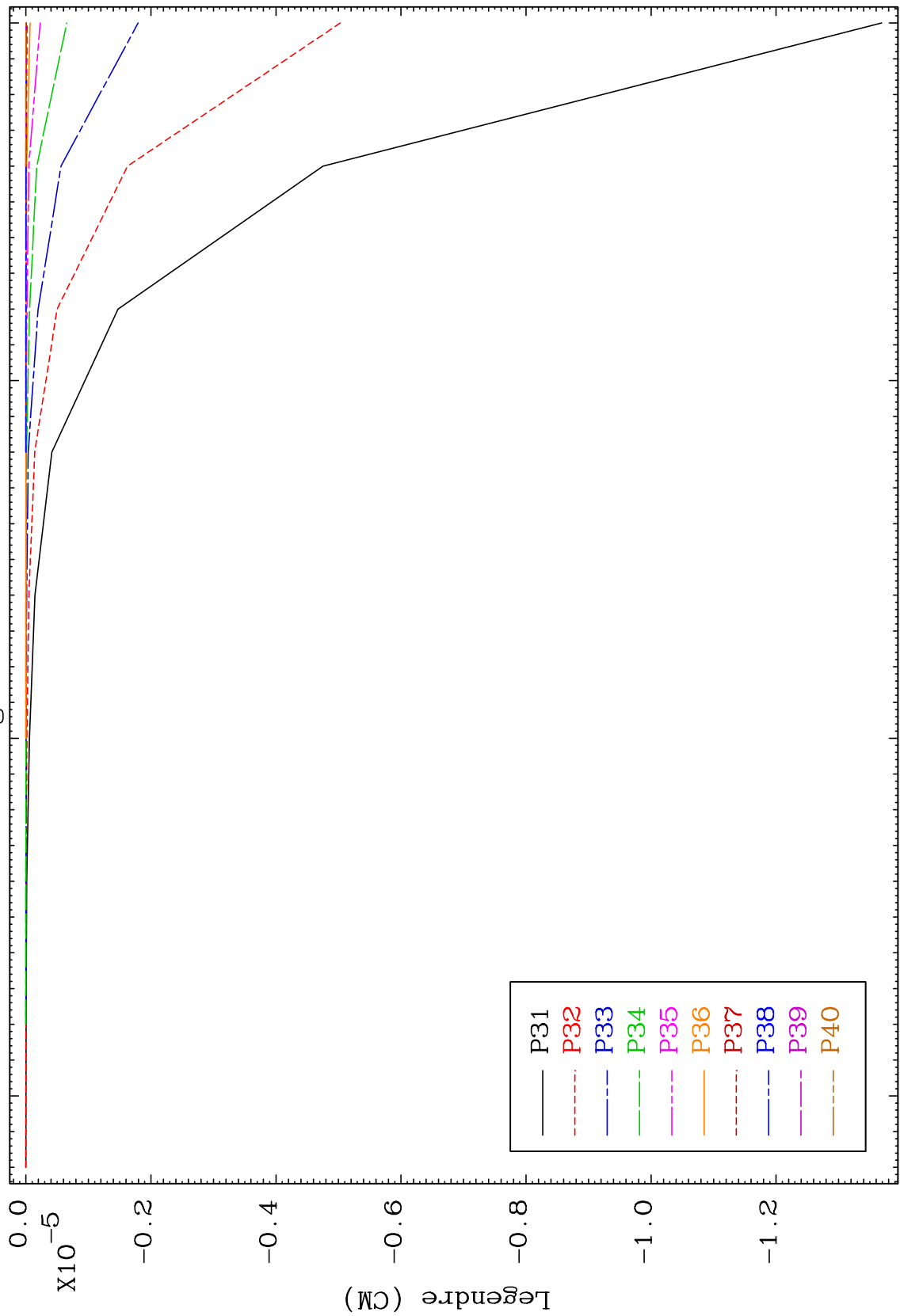
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

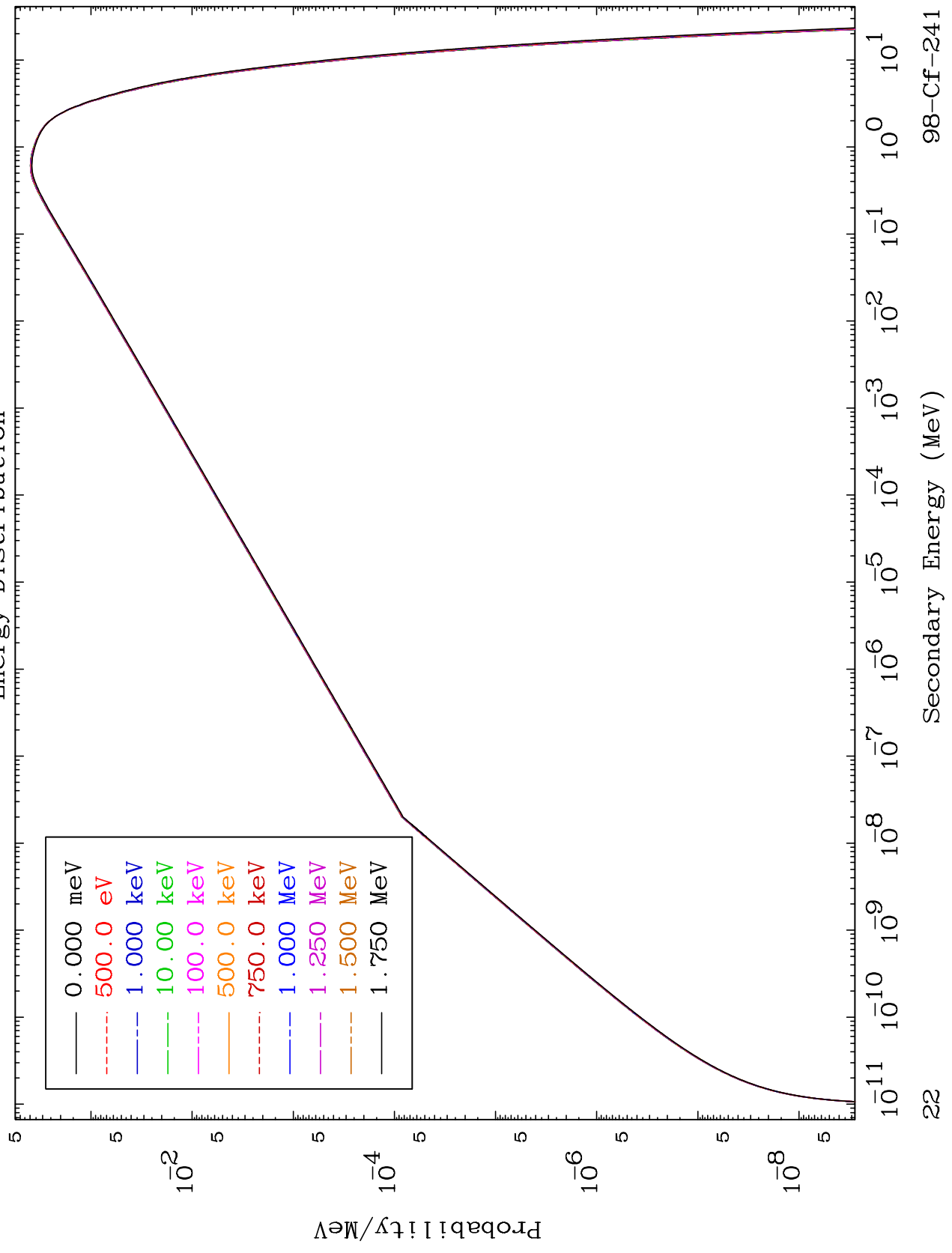
98-Cf-241



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### Fission Energy Distribution

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98-Cf-241

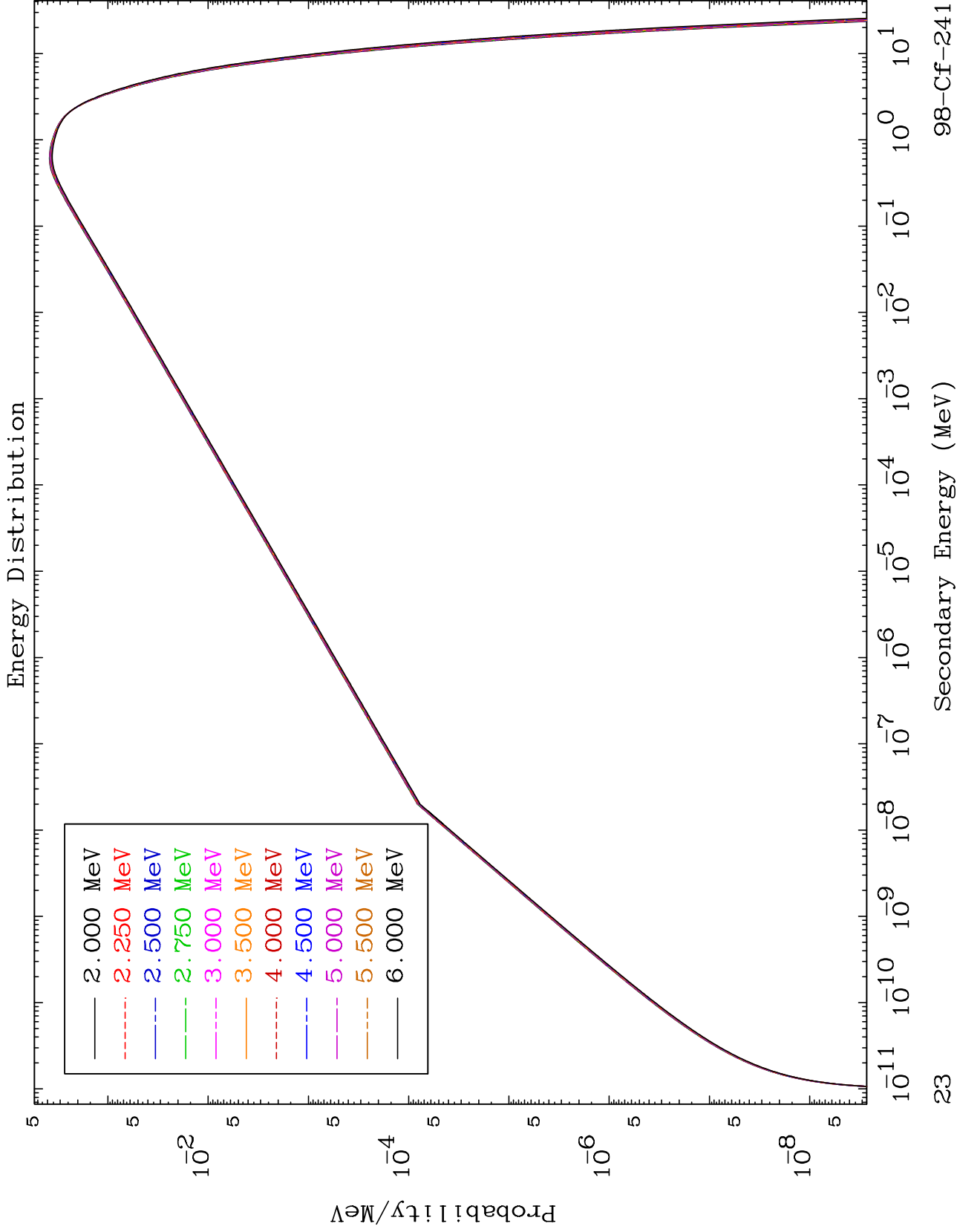
Secondary Energy (MeV)

22

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Fission

98-Cf-241

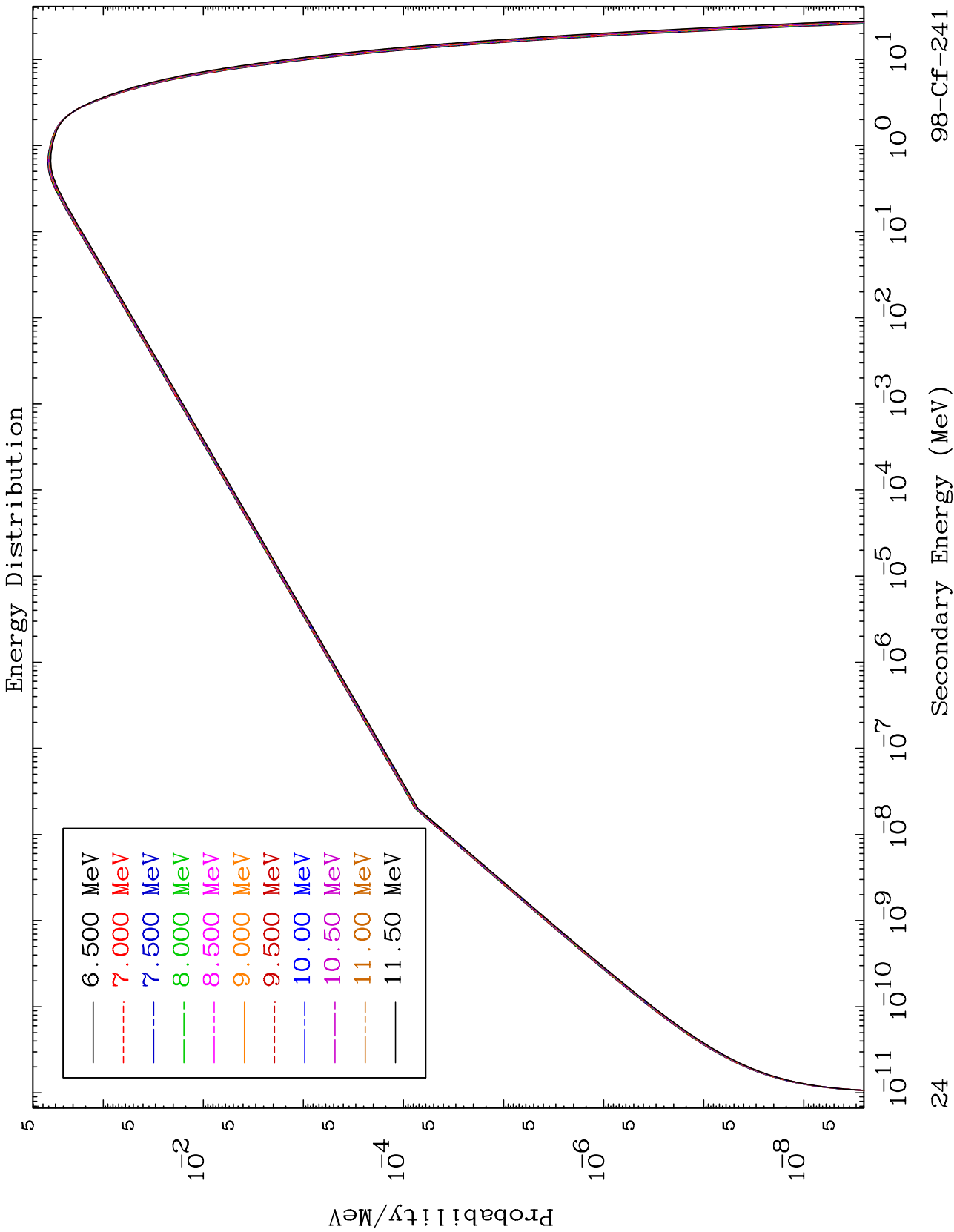




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Fission

98-Cf-241

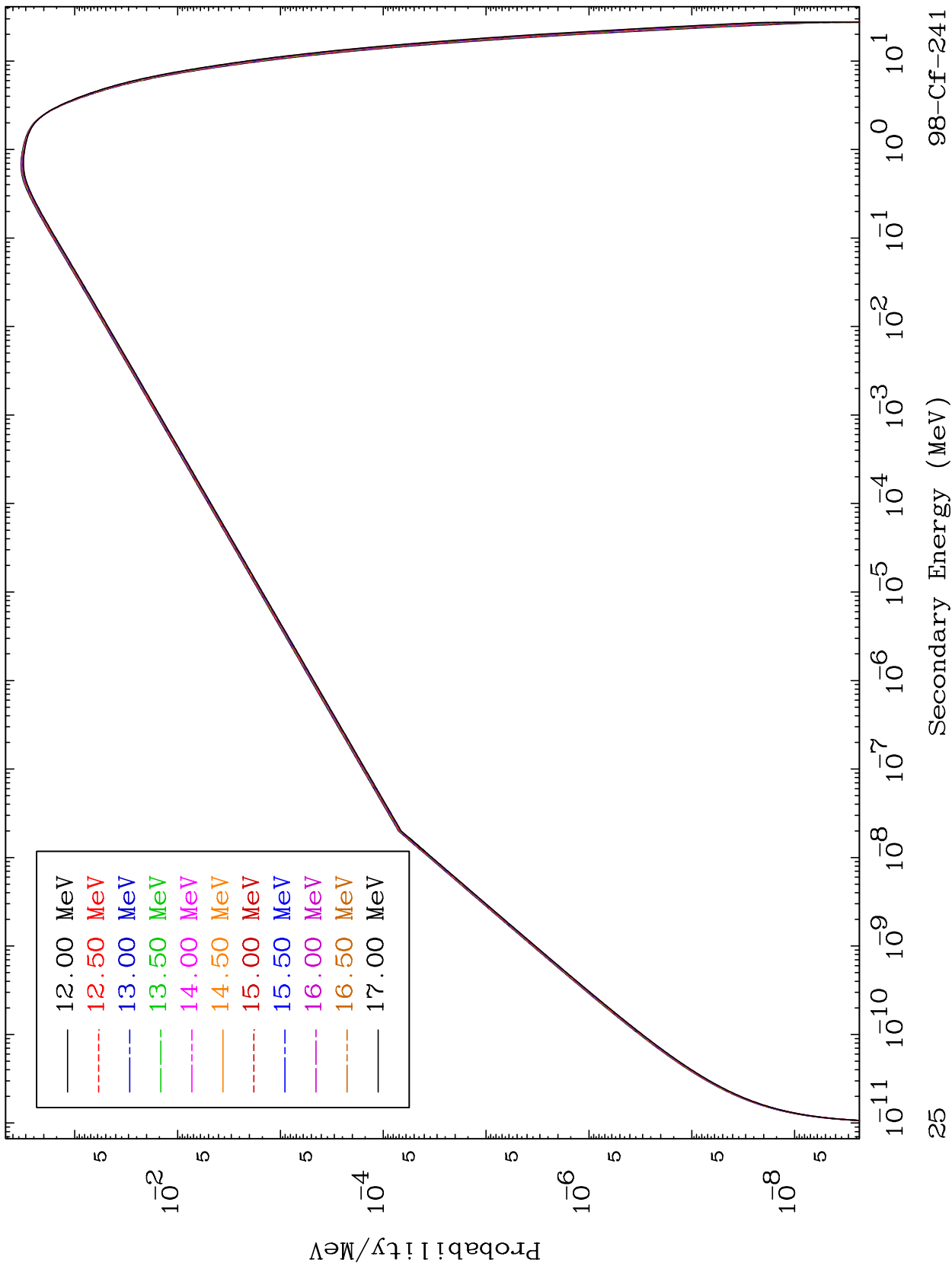


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### Fission Energy Distribution

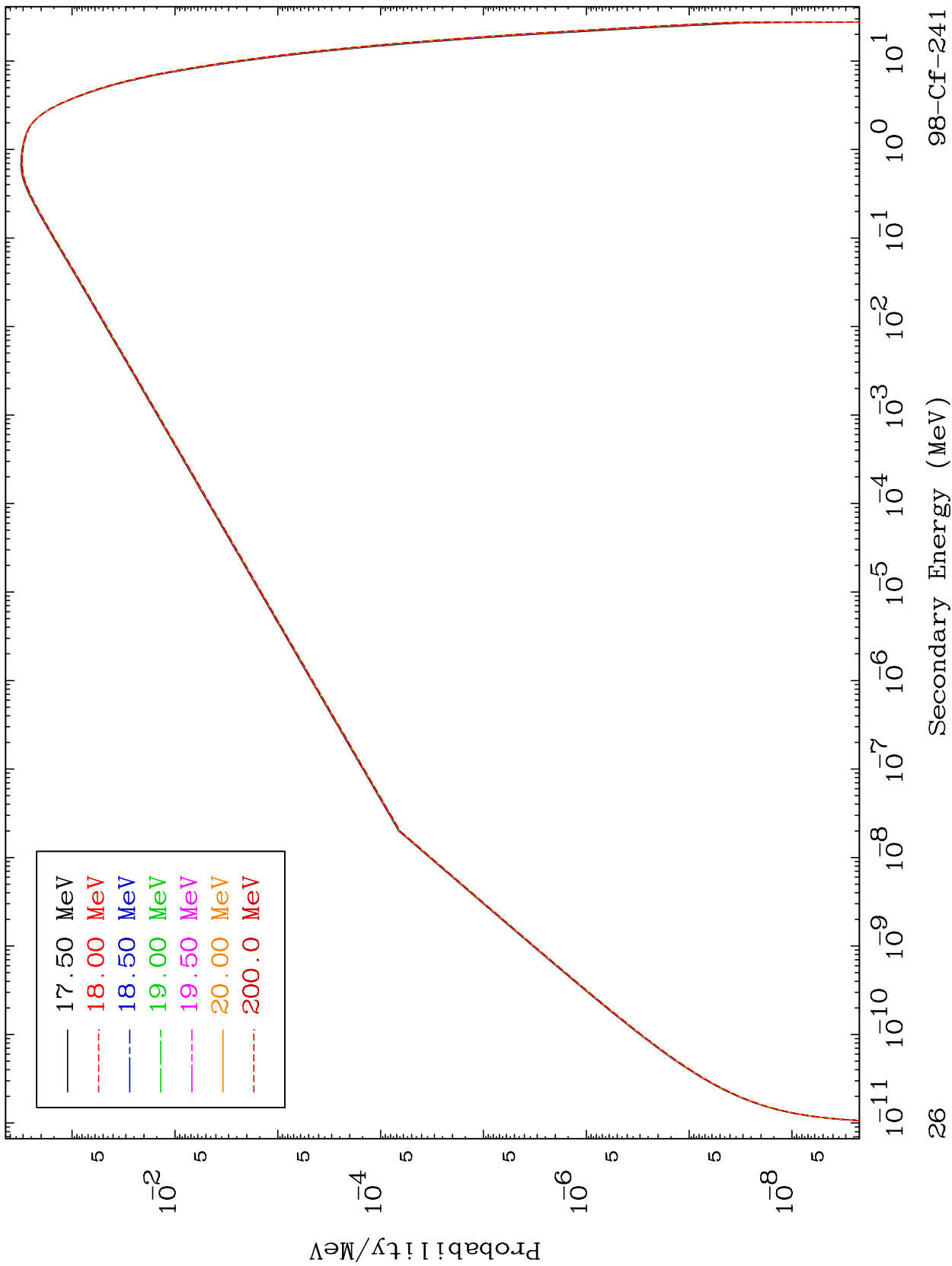
98-Cf-241



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Fission Energy Distribution

98-Cf-241

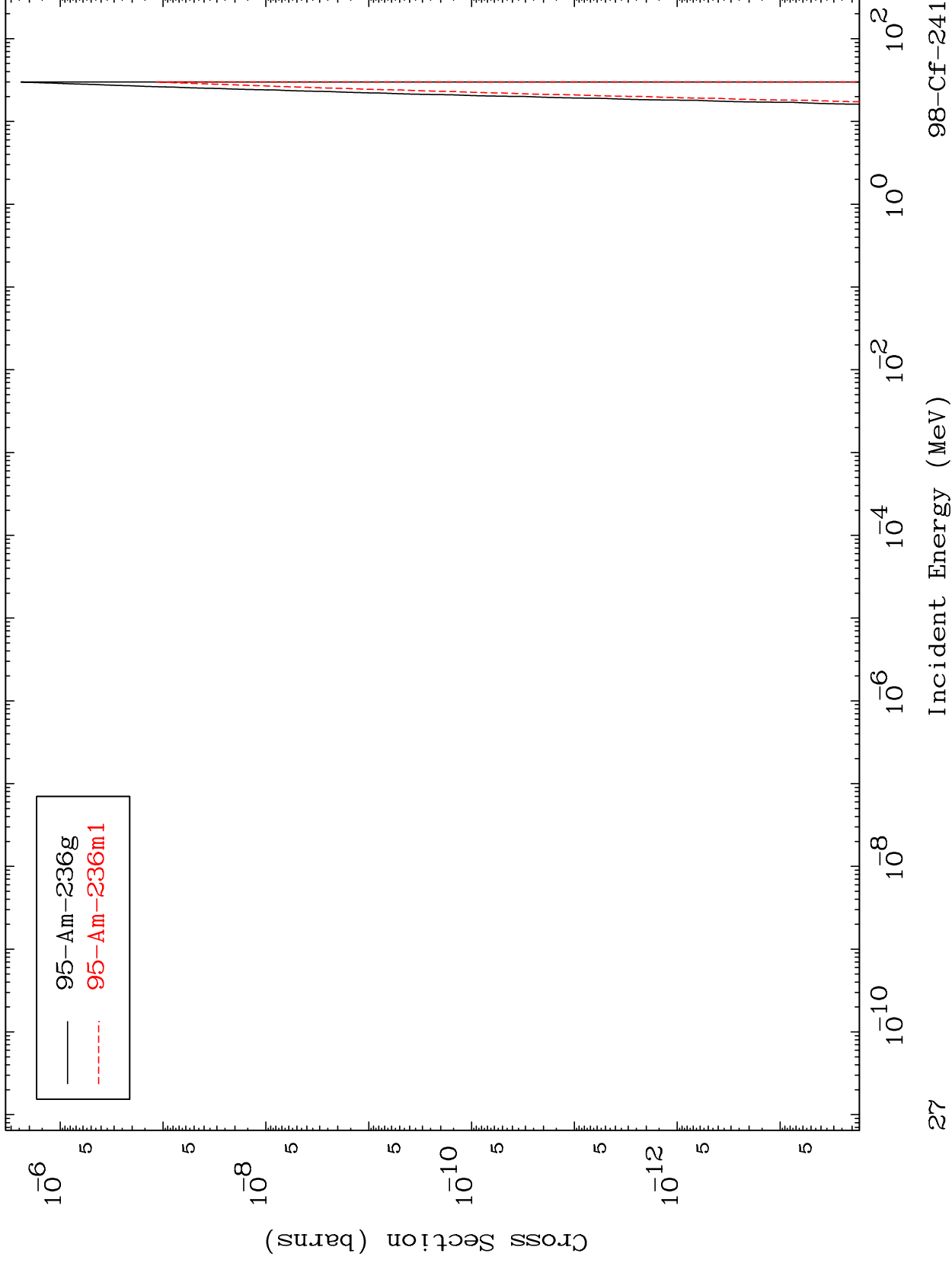


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(n,n') p  $\alpha$

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Radionuclide Production Cross Section



MAT 9828

(n,d)  $\alpha$

98-Cf-241

Radionuclide Production Cross Section

