

Program EVALPLOT  
(Version 2018-1)

by

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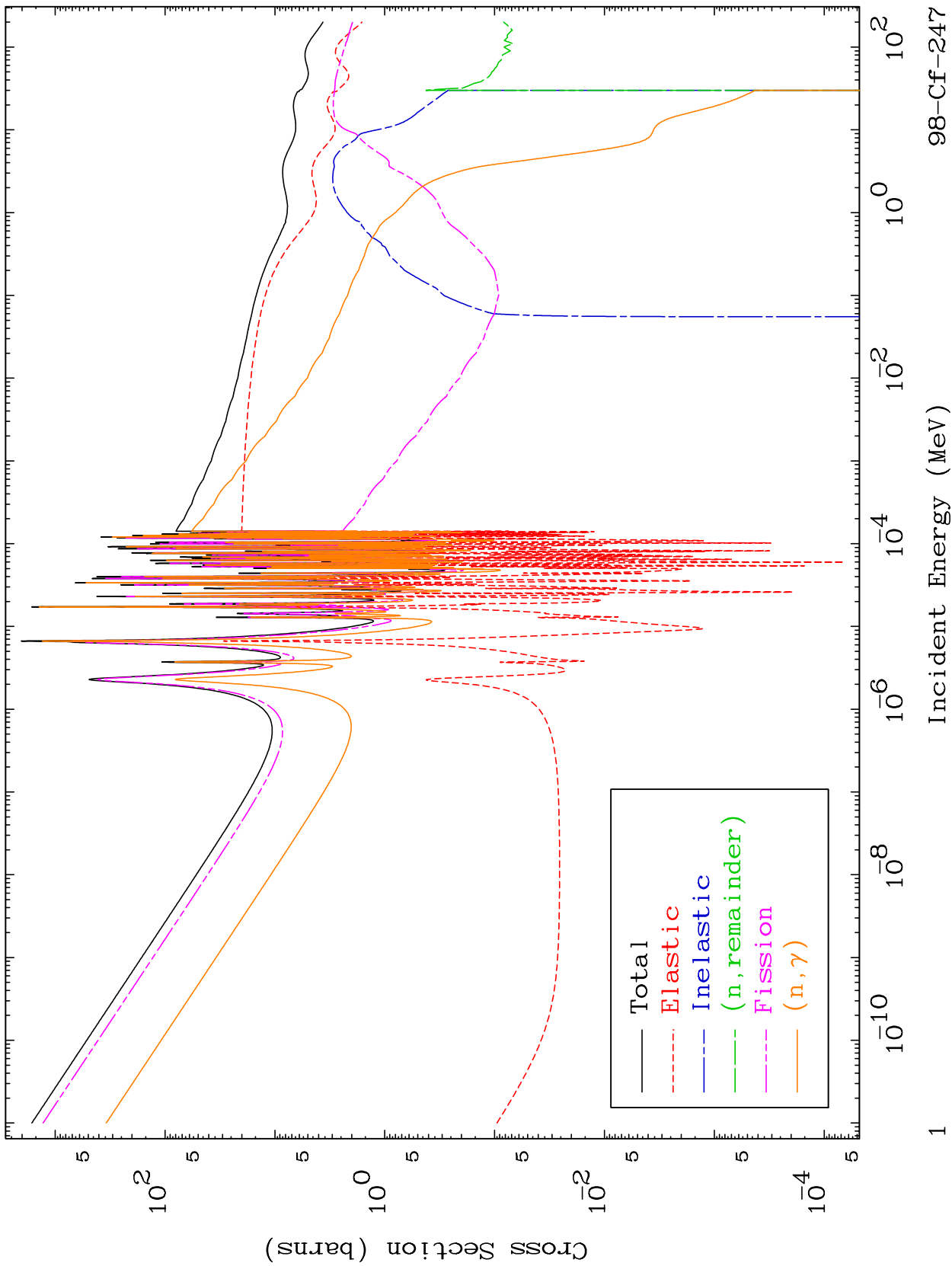
Press Mouse Button to Start

MAT 9846

Major

293 Kelvin Cross Sections

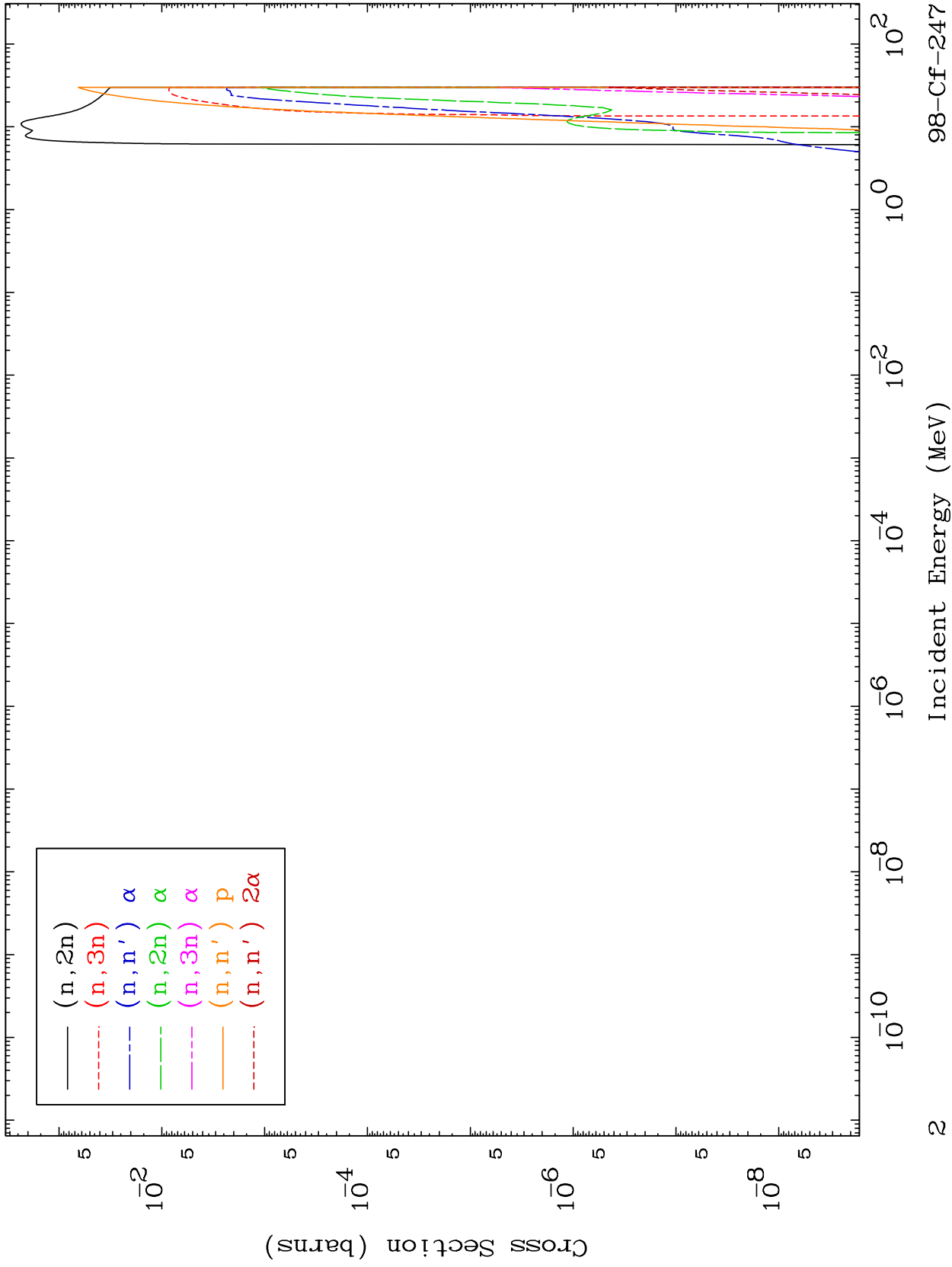
98-Cf-247



MAT 9846

Neutron Production  
293 Kelvin Cross Sections

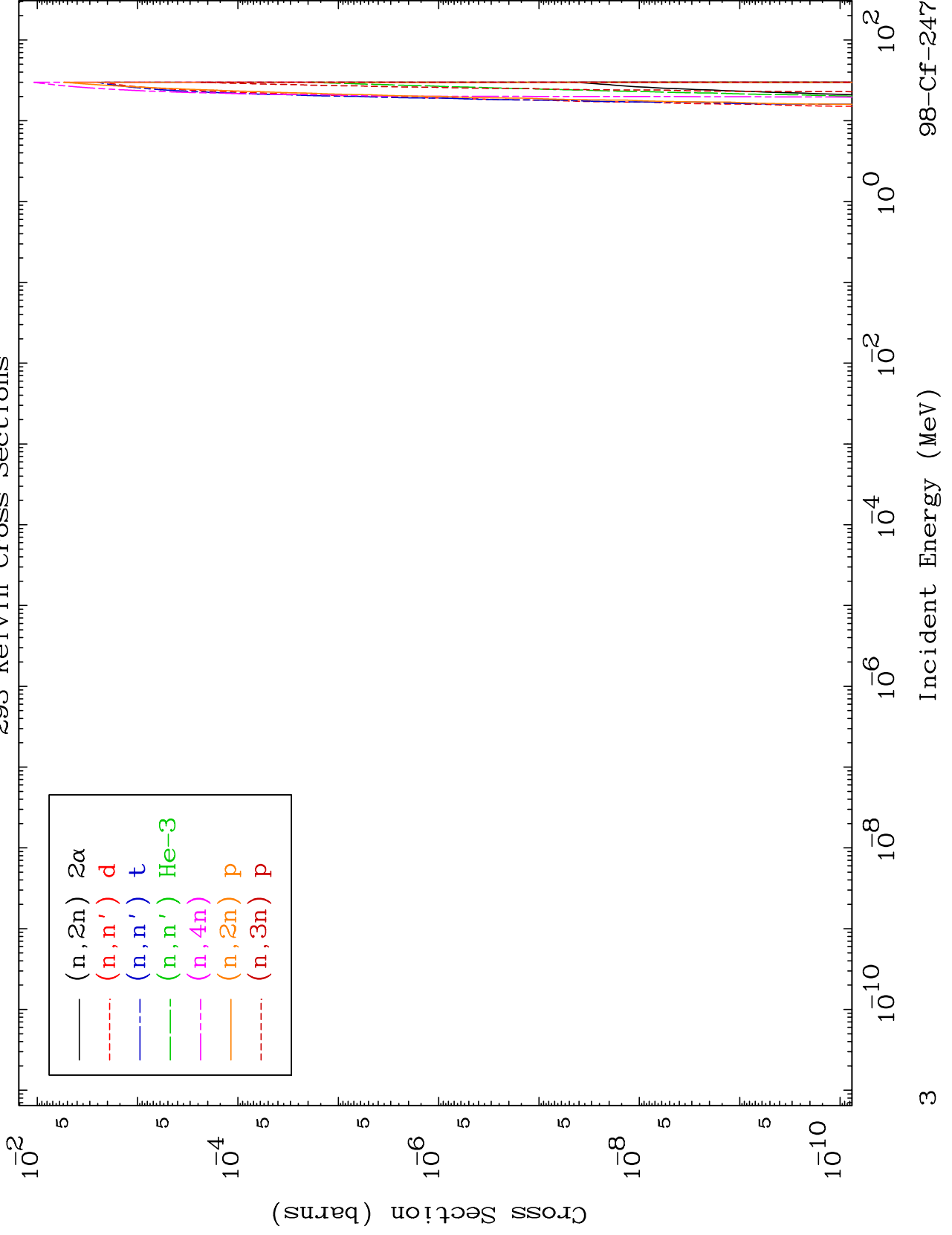
98-Cf-247



MAT 9846

Neutron Production  
293 Kelvin Cross Sections

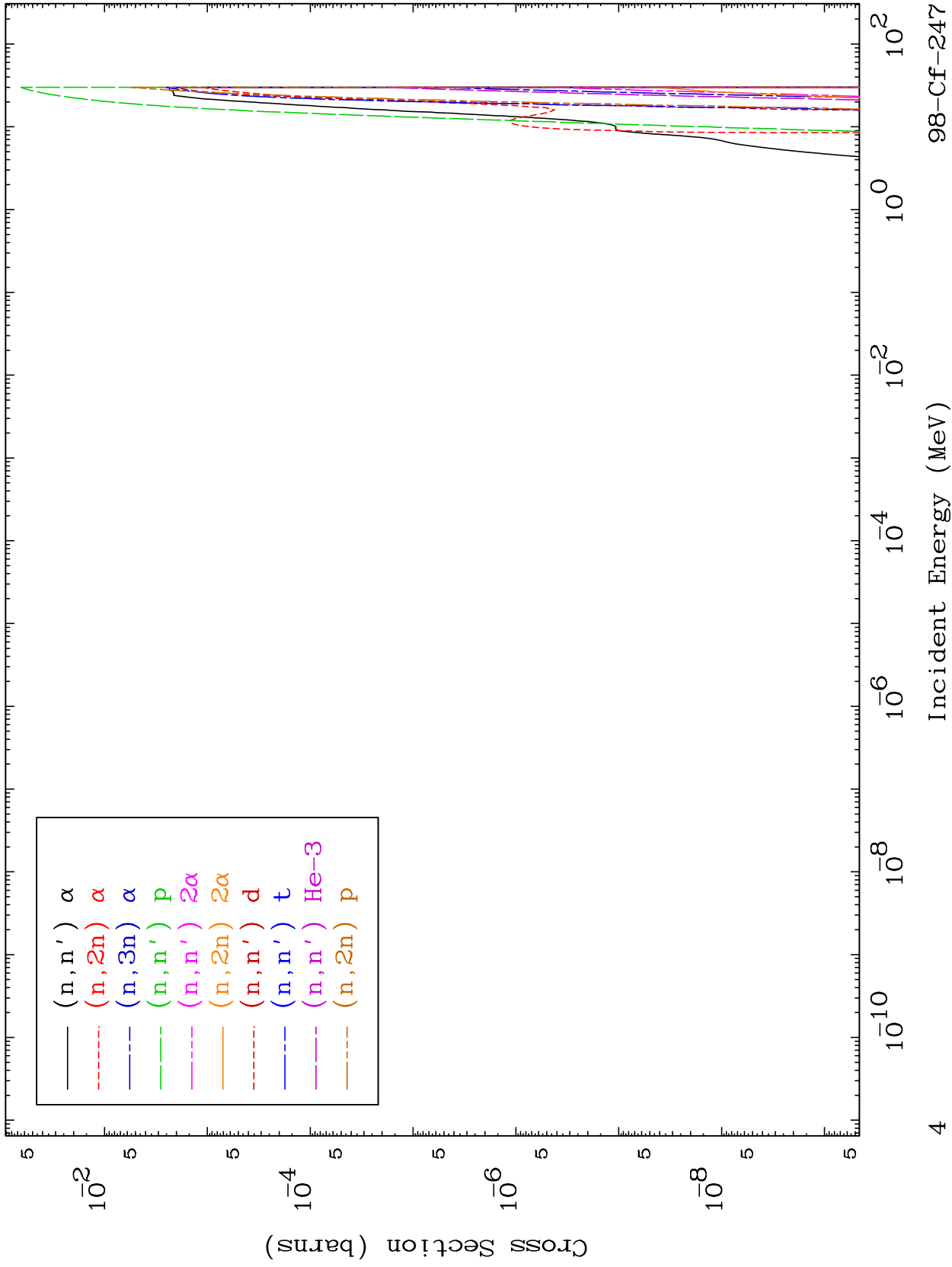
98-Cf-247



MAT 9846

Charged Particle  
293 Kelvin Cross Sections

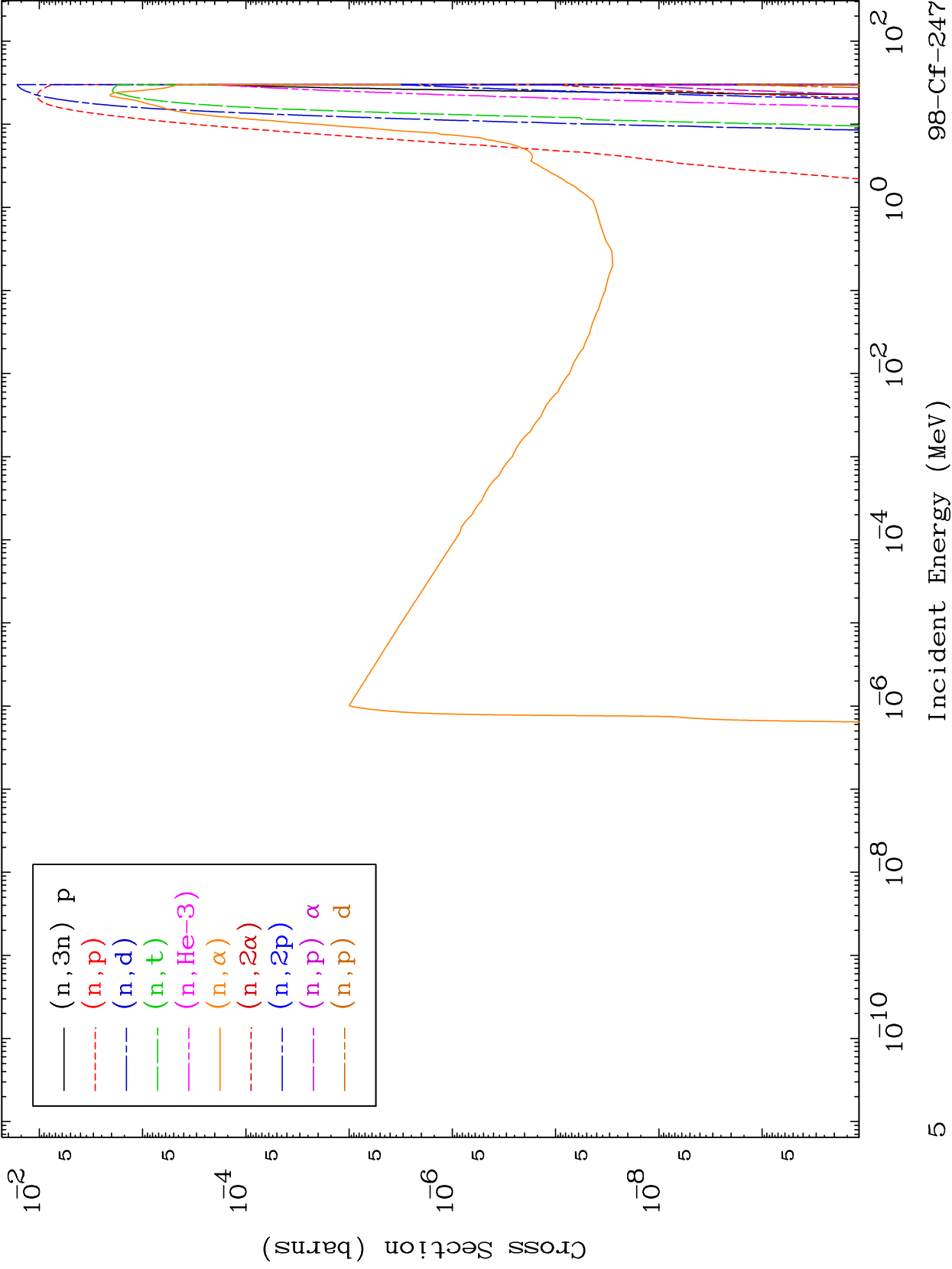
98-Cf-247



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Charged Particle  
293 Kelvin Cross Sections

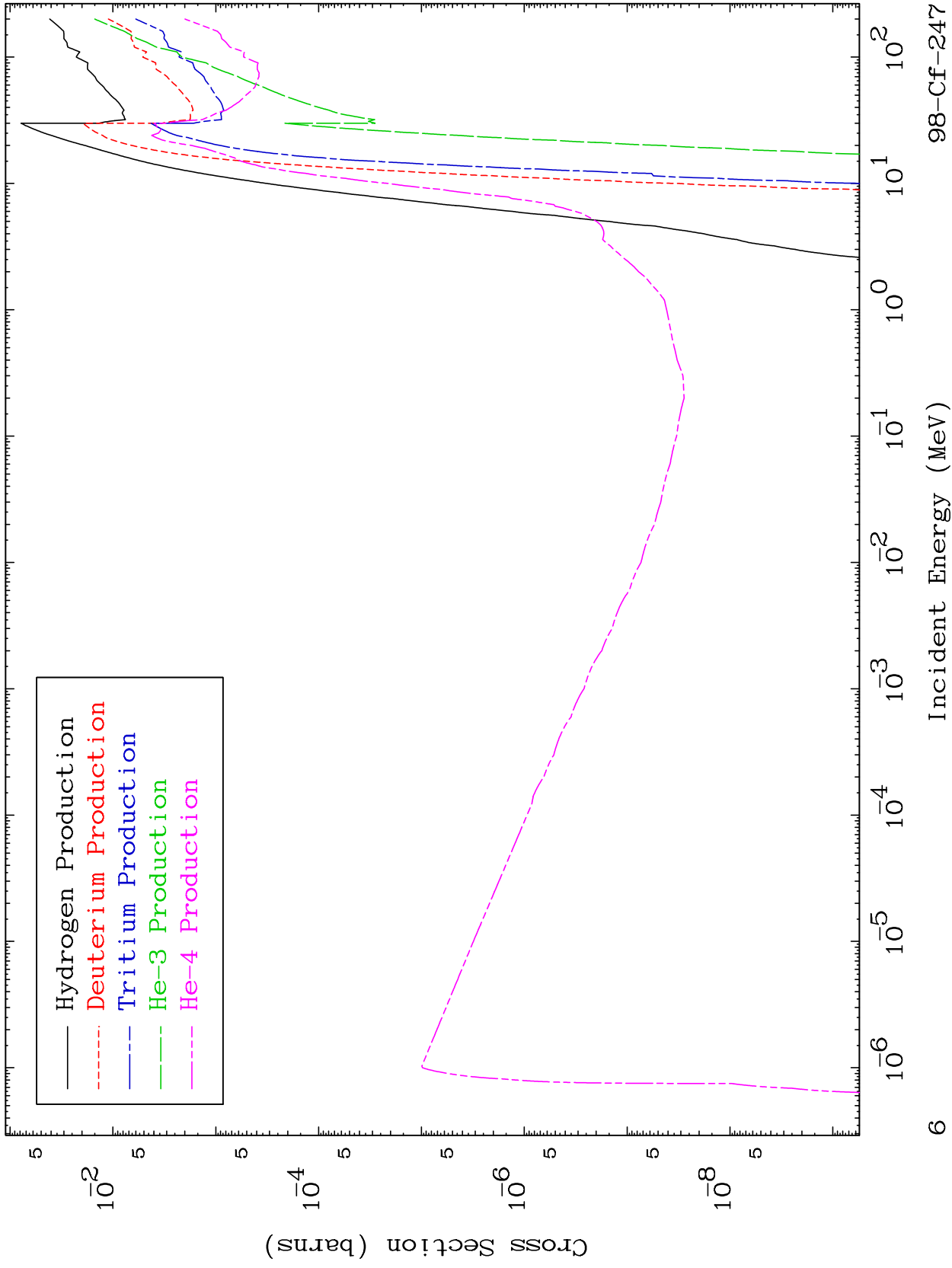
98-Cf-247



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Particle Production  
293 Kelvin Cross Sections

98-Cf-247



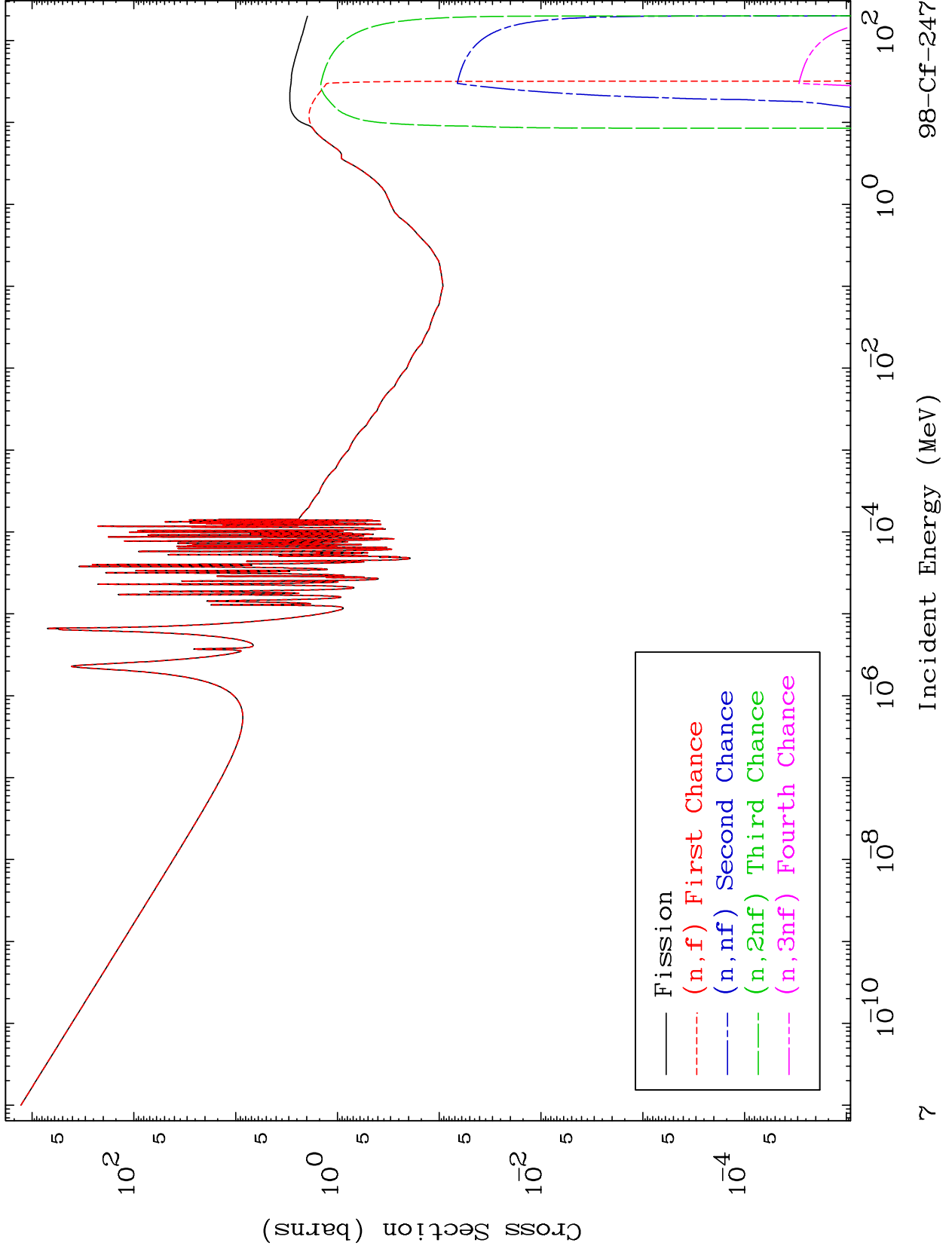
6

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Fission  
293 Kelvin Cross Sections

98-Cf-247



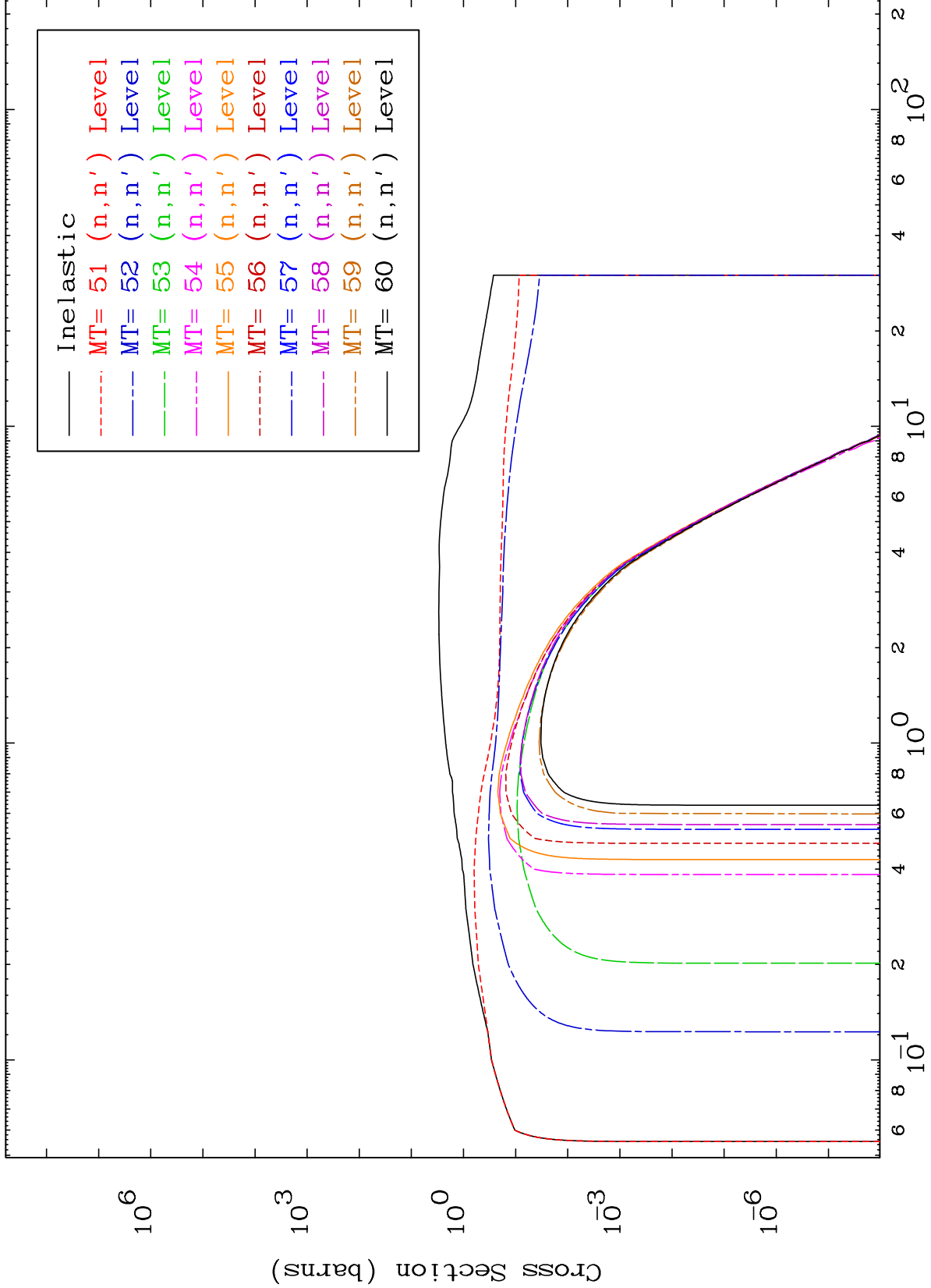


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(n,n') Level

98-Cf-247

293 Kelvin Cross Sections

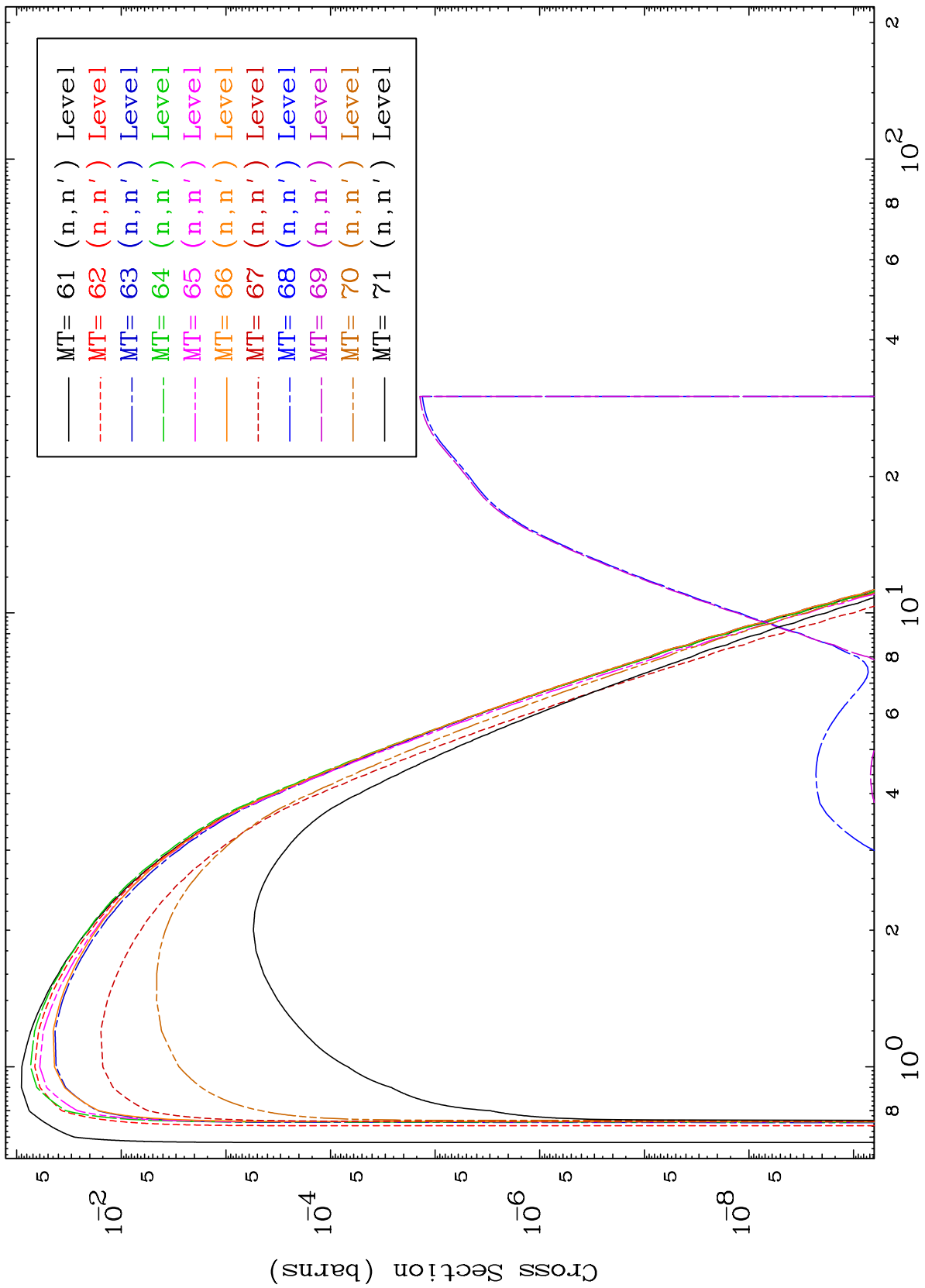


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(n,n') Level

98-Cf-247

293 Kelvin Cross Sections



9

Incident Energy (MeV)

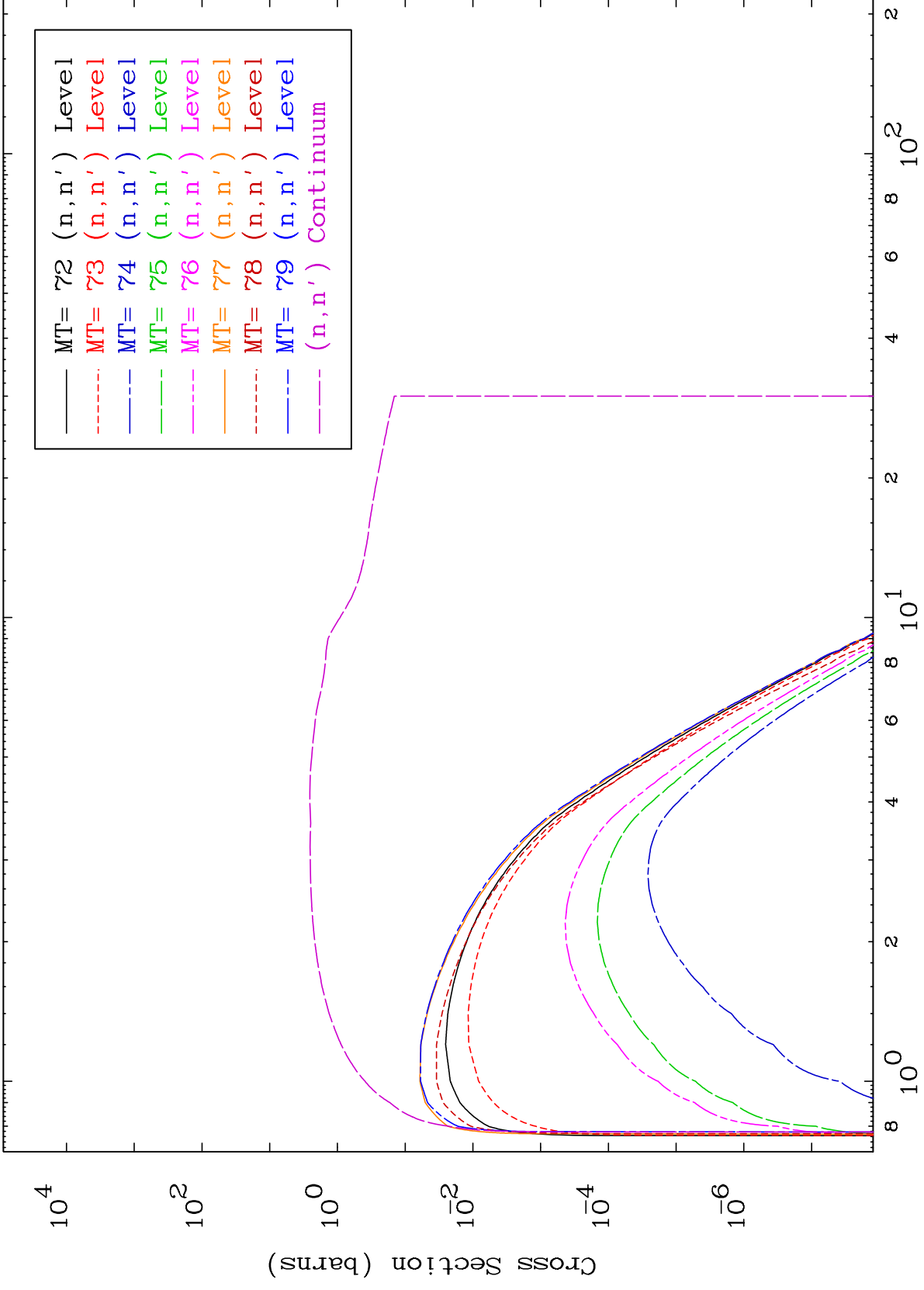
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(n,n') Level

98-Cf-247

293 Kelvin Cross Sections



98-Cf-247

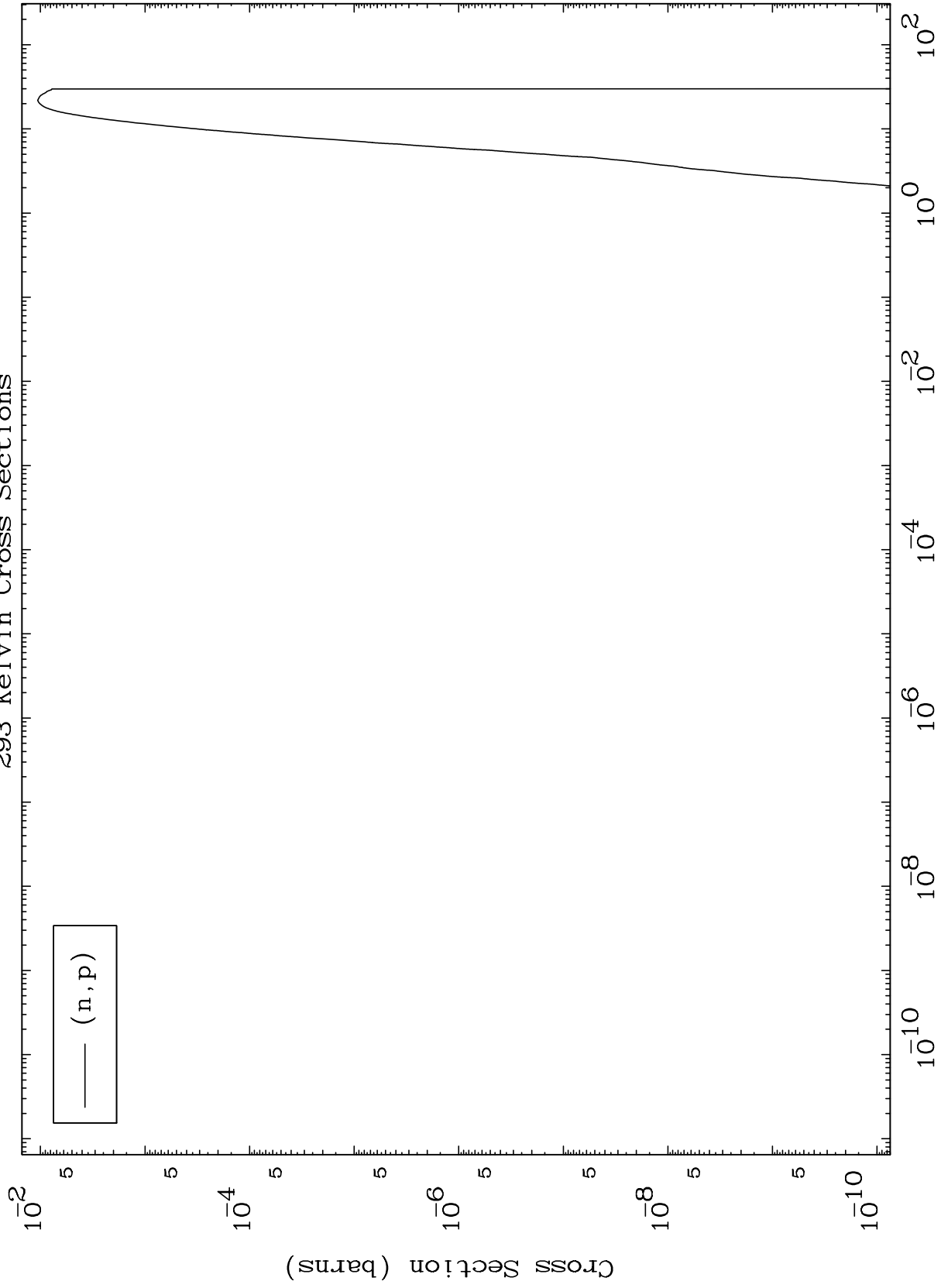
Incident Energy (MeV)

10

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(n,p) Levels  
293 Kelvin Cross Sections

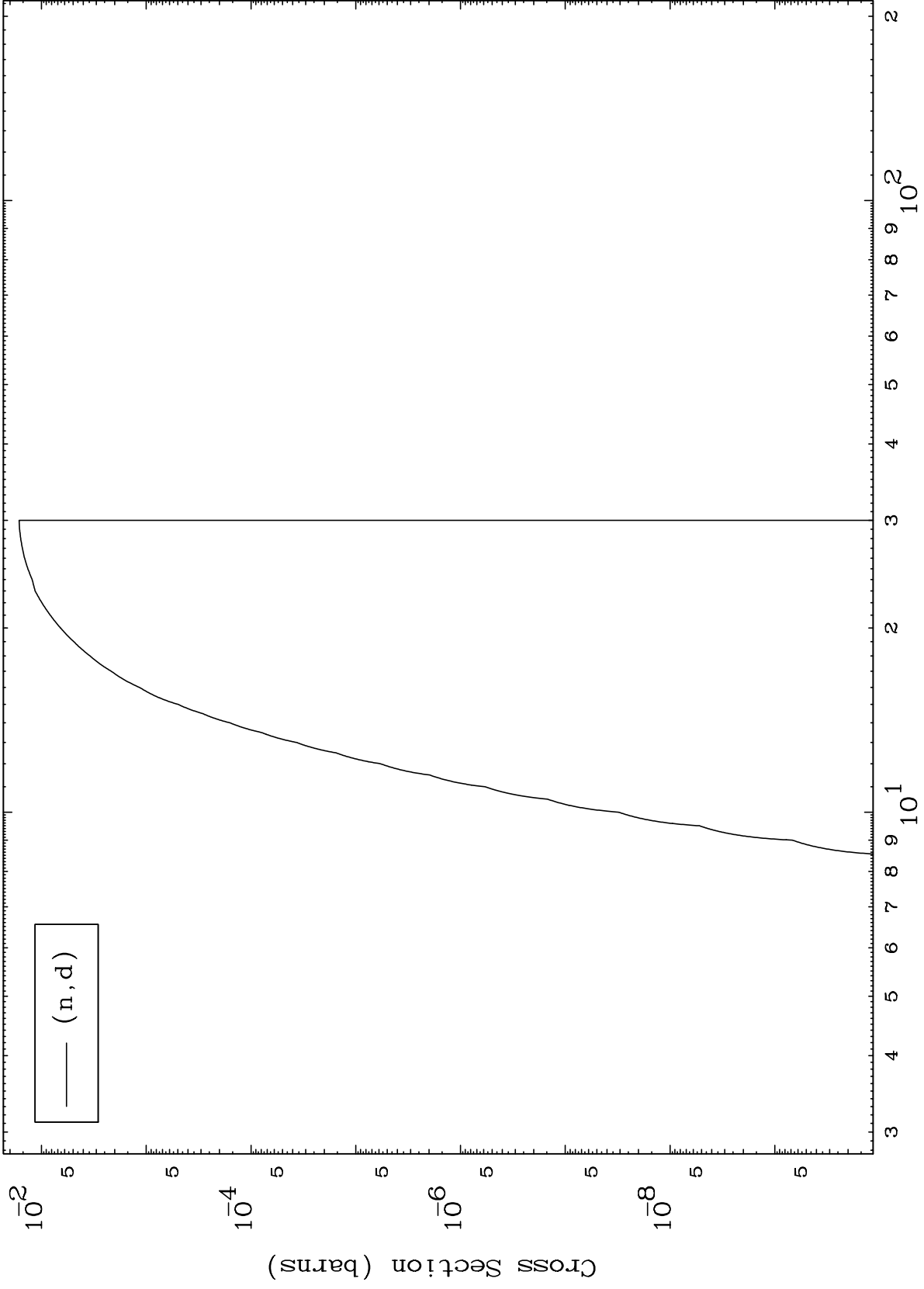
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(n,d) Levels  
293 Kelvin Cross Sections

98-Cf-247



12

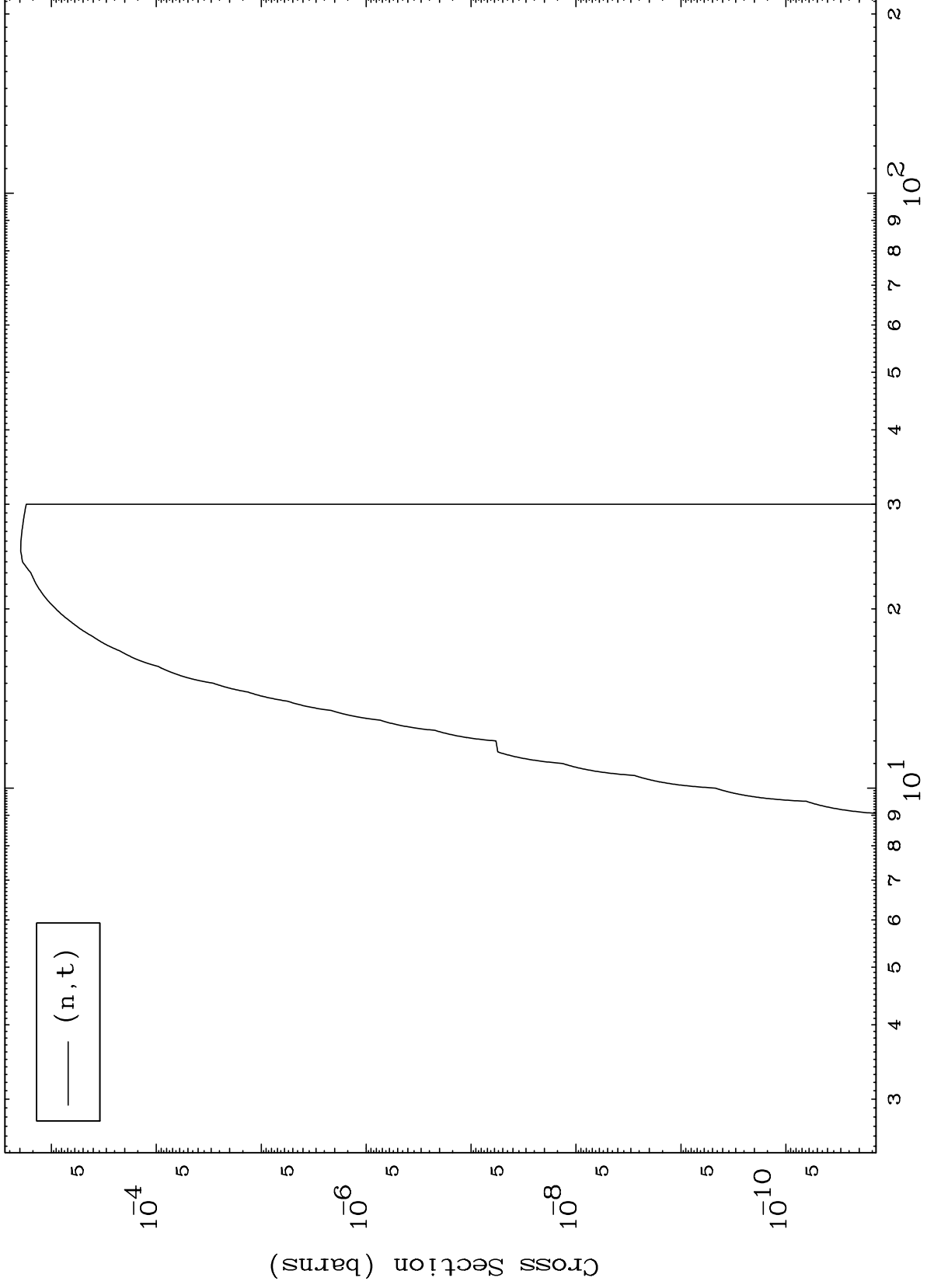
Incident Energy (MeV)

98-Cf-247

MAT 9846

(n,t) Levels  
293 Kelvin Cross Sections

98-Cf-247



13

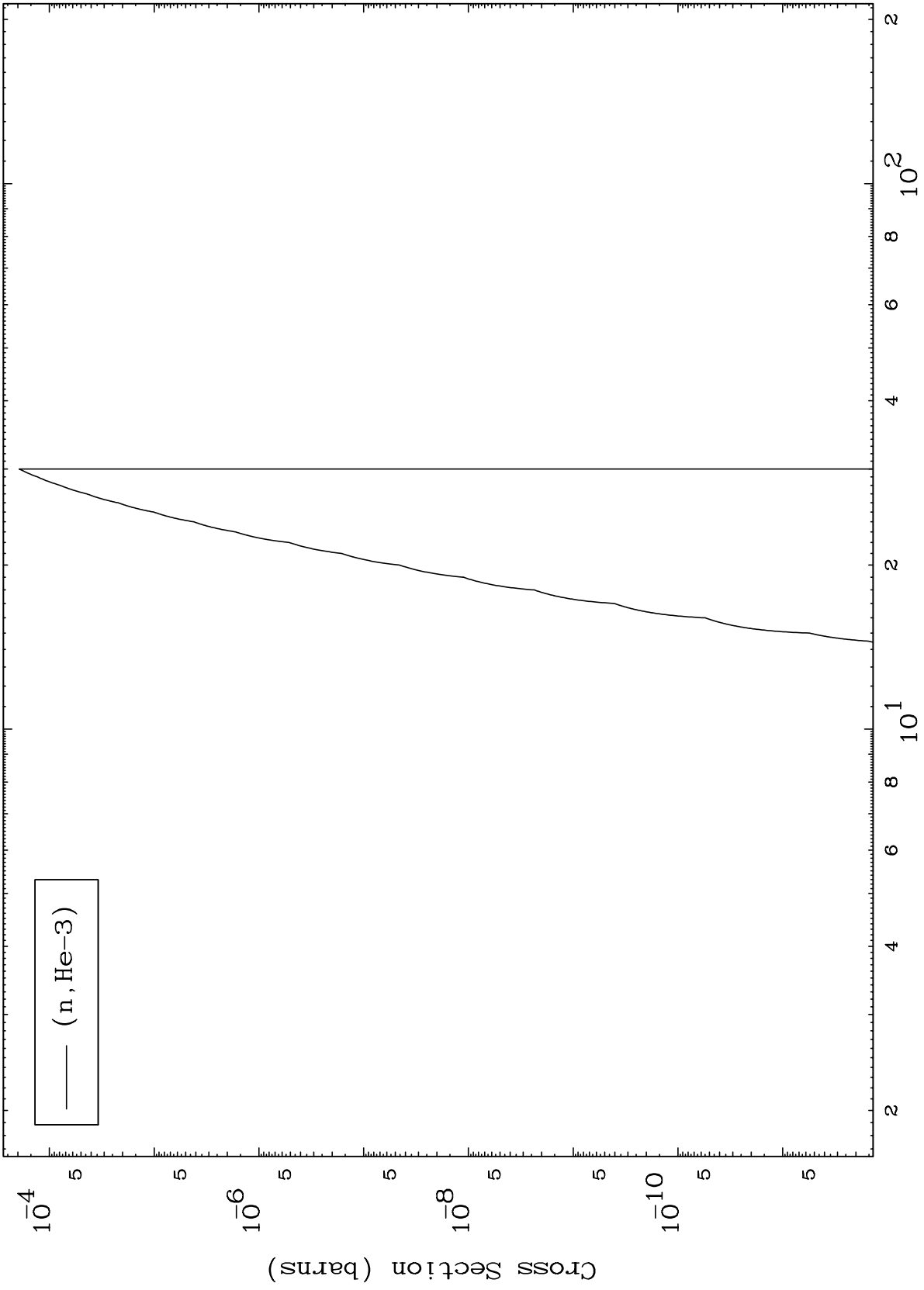
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

98-Cf-247



14

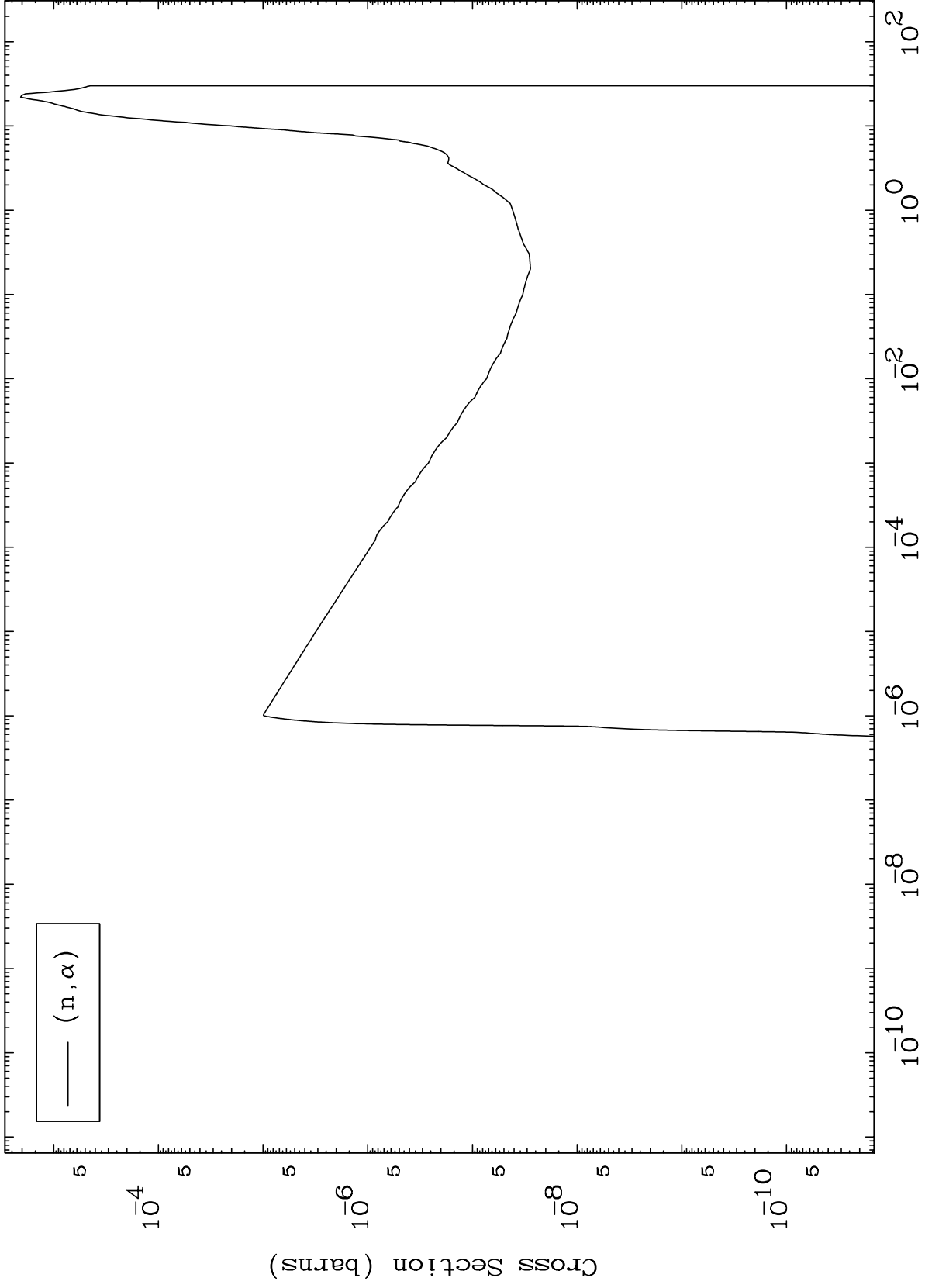
Incident Energy (MeV)

98-Cf-247

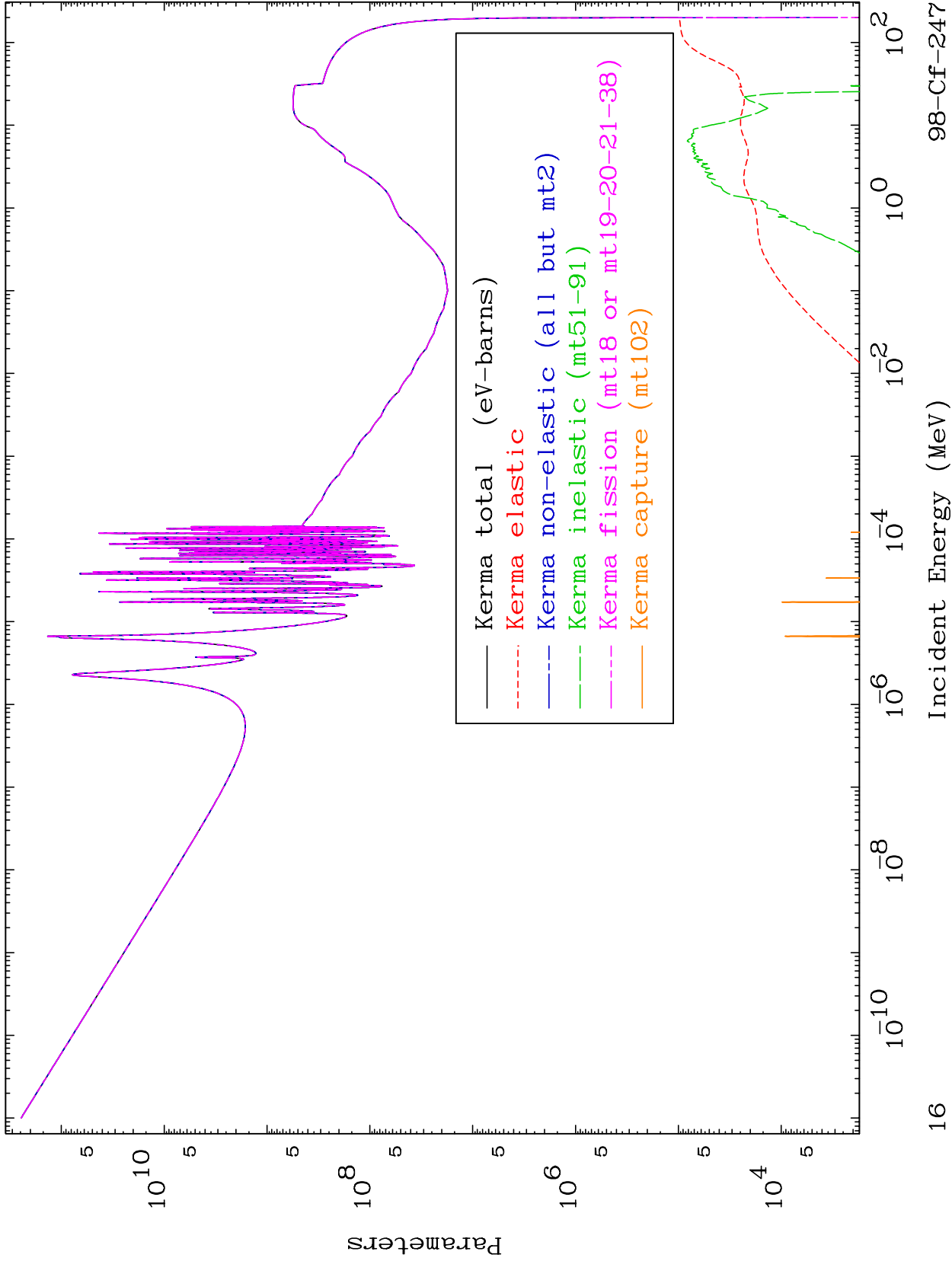
MAT 9846

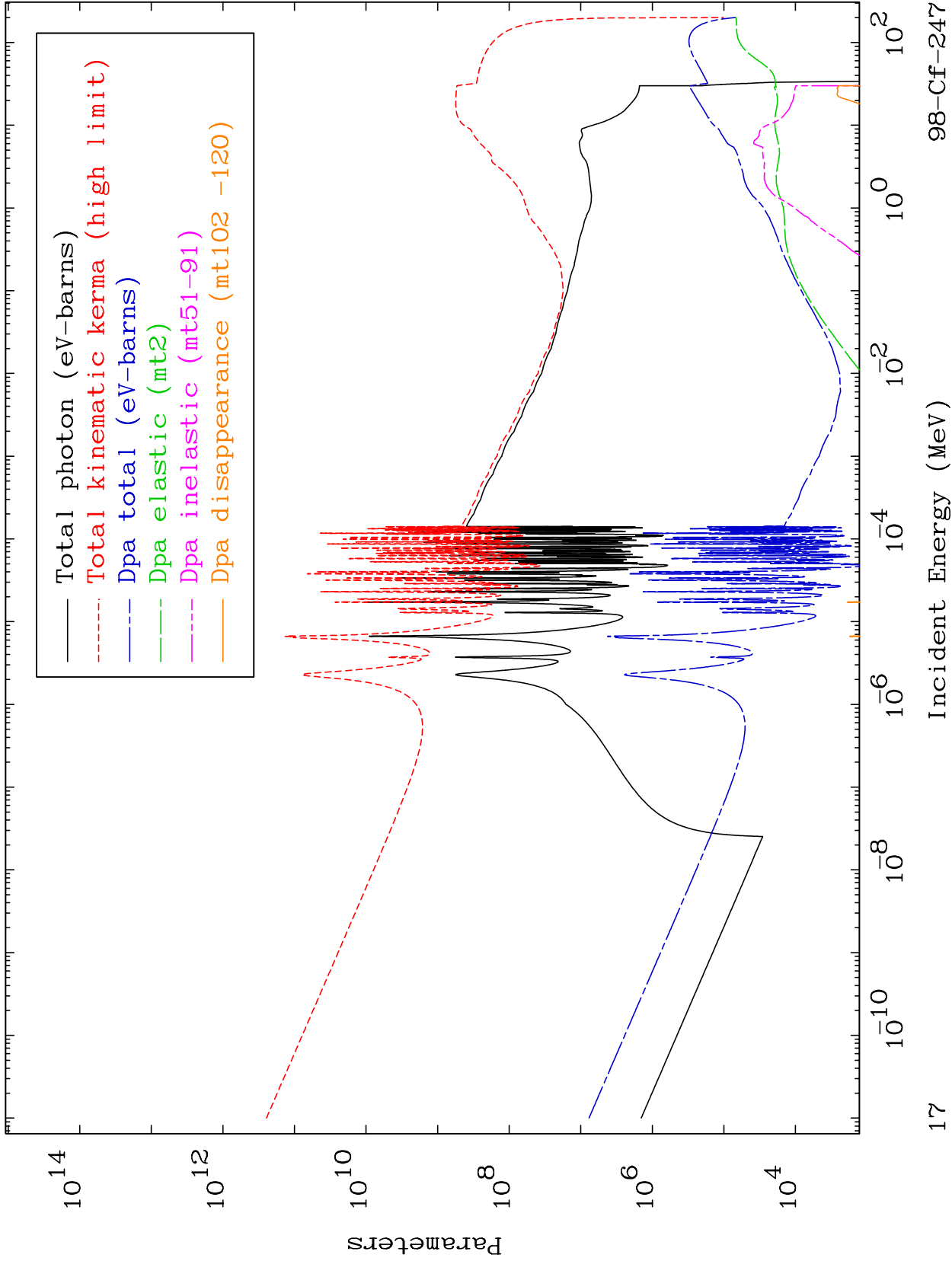
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

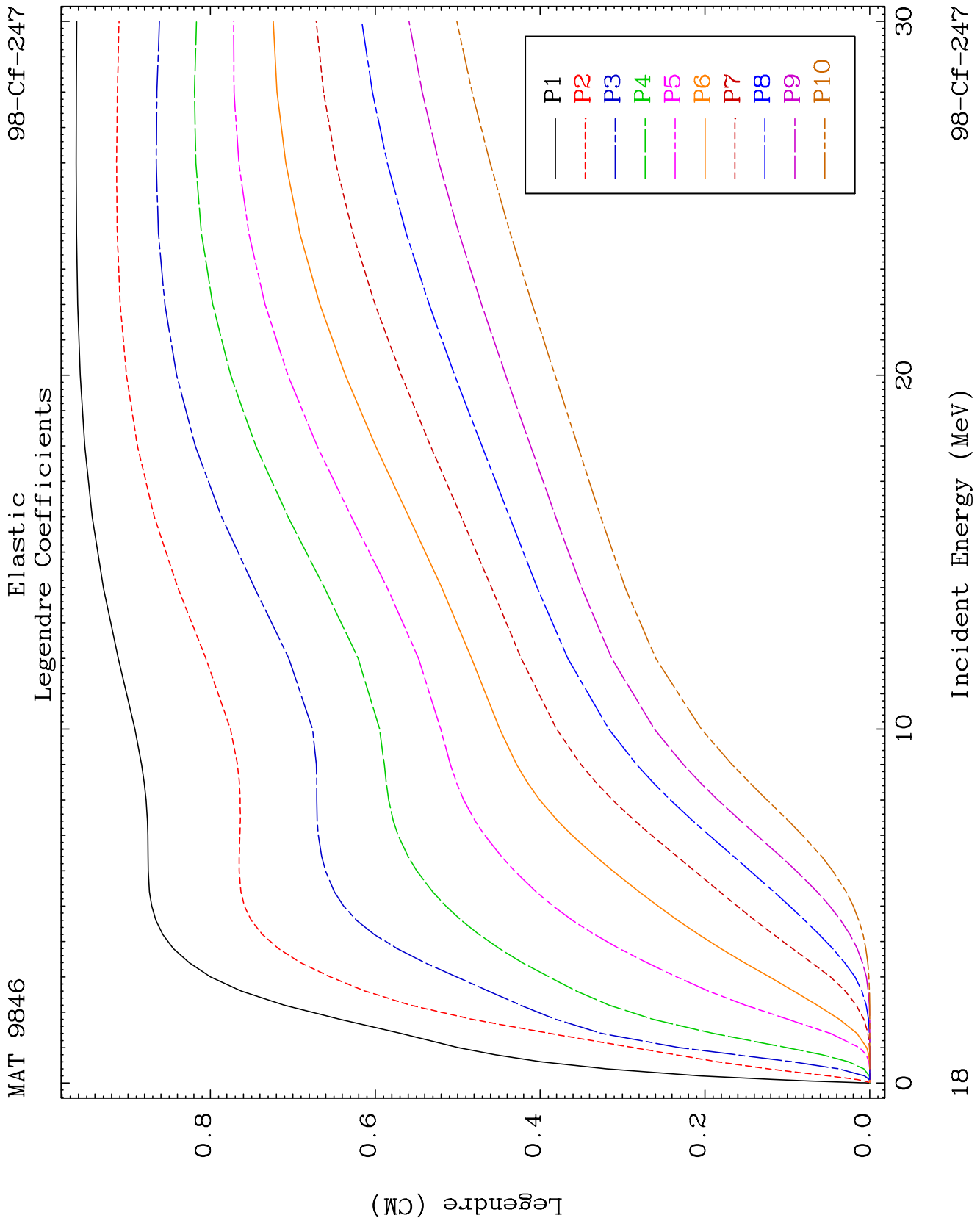
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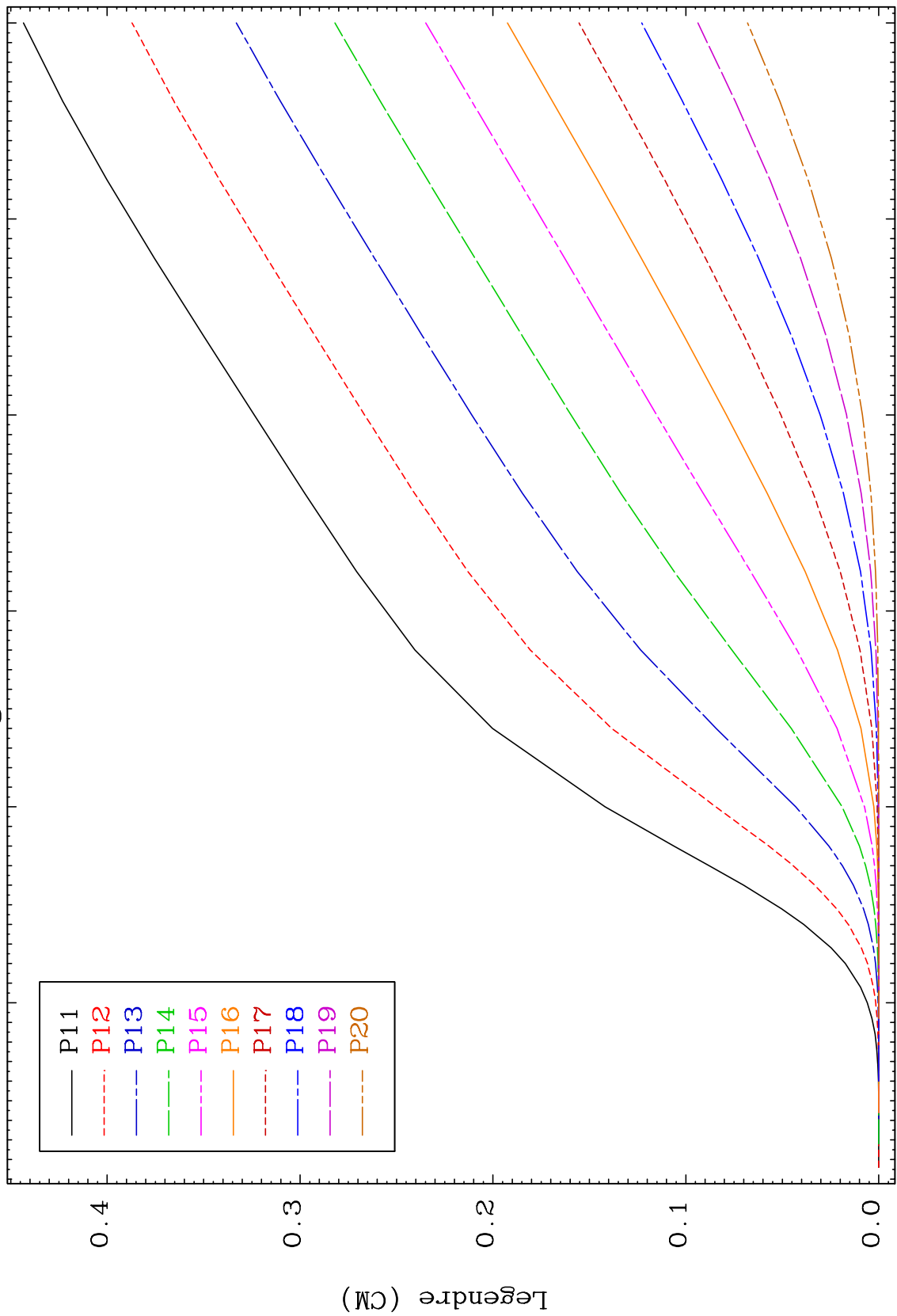




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Elastic Legendre Coefficients

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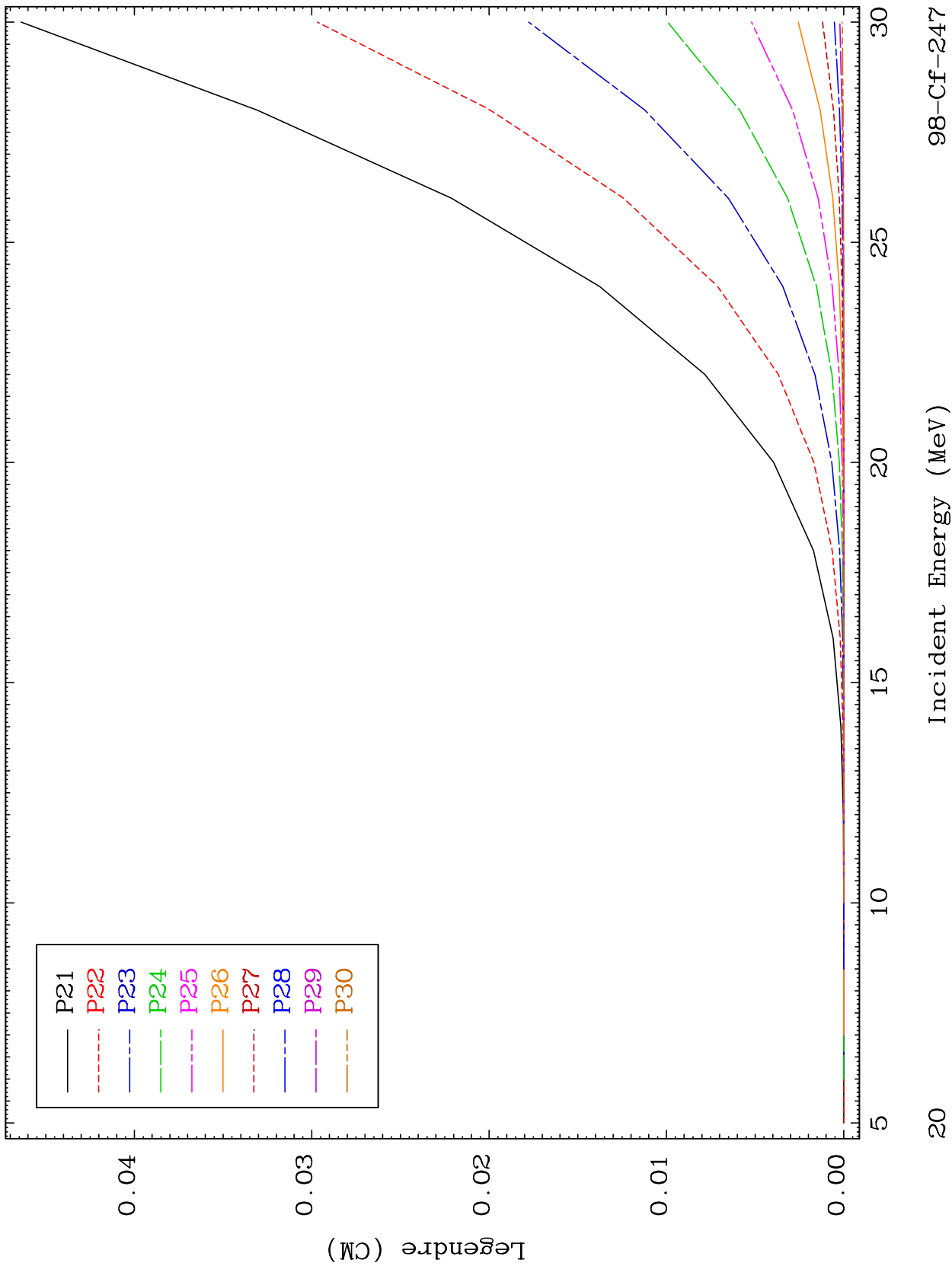
98-Cf-247

Incident Energy (MeV)

MAT 9846

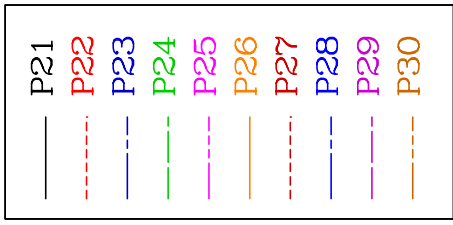
### Elastic Legendre Coefficients

98-Cf-247



98-Cf-247

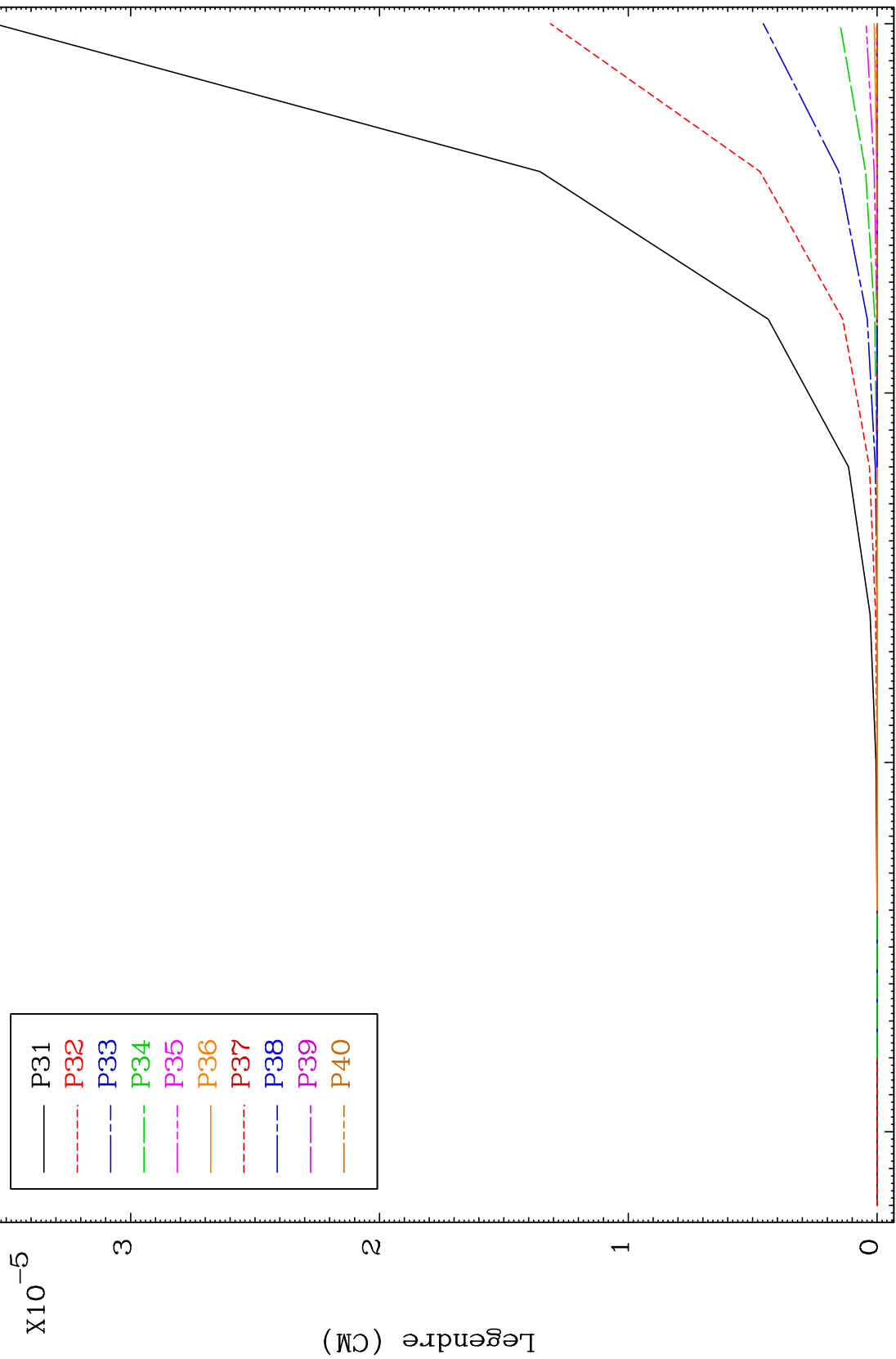
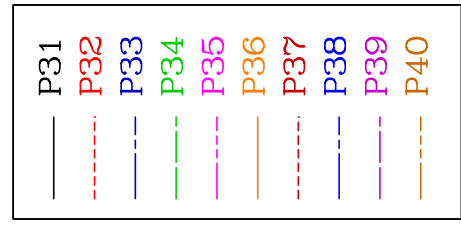
Incident Energy (MeV)



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Elastic Legendre Coefficients

98-Cf-247



21

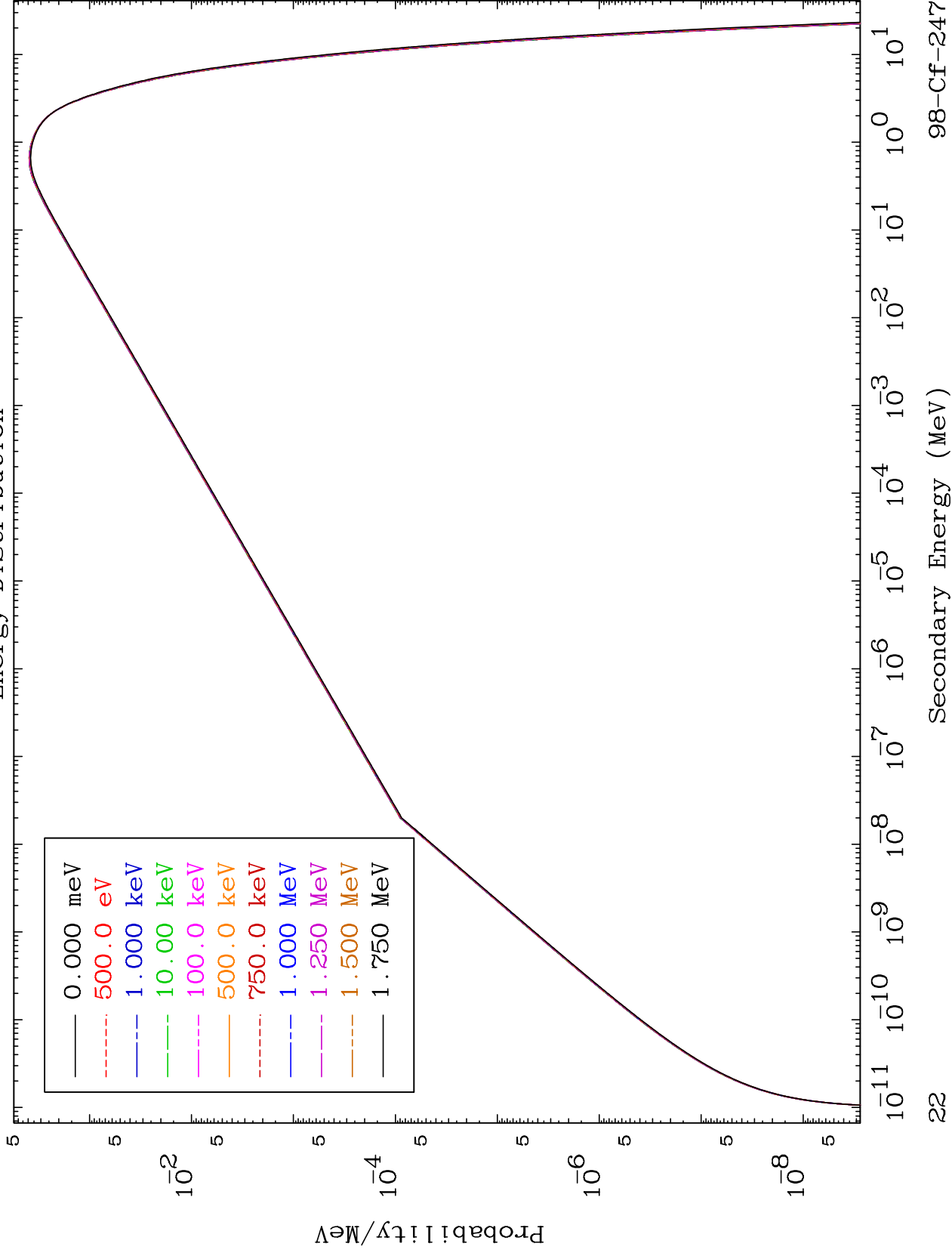
Incident Energy (MeV)

98-Cf-247

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### Fission Energy Distribution

98-Cf-247



98-Cf-247

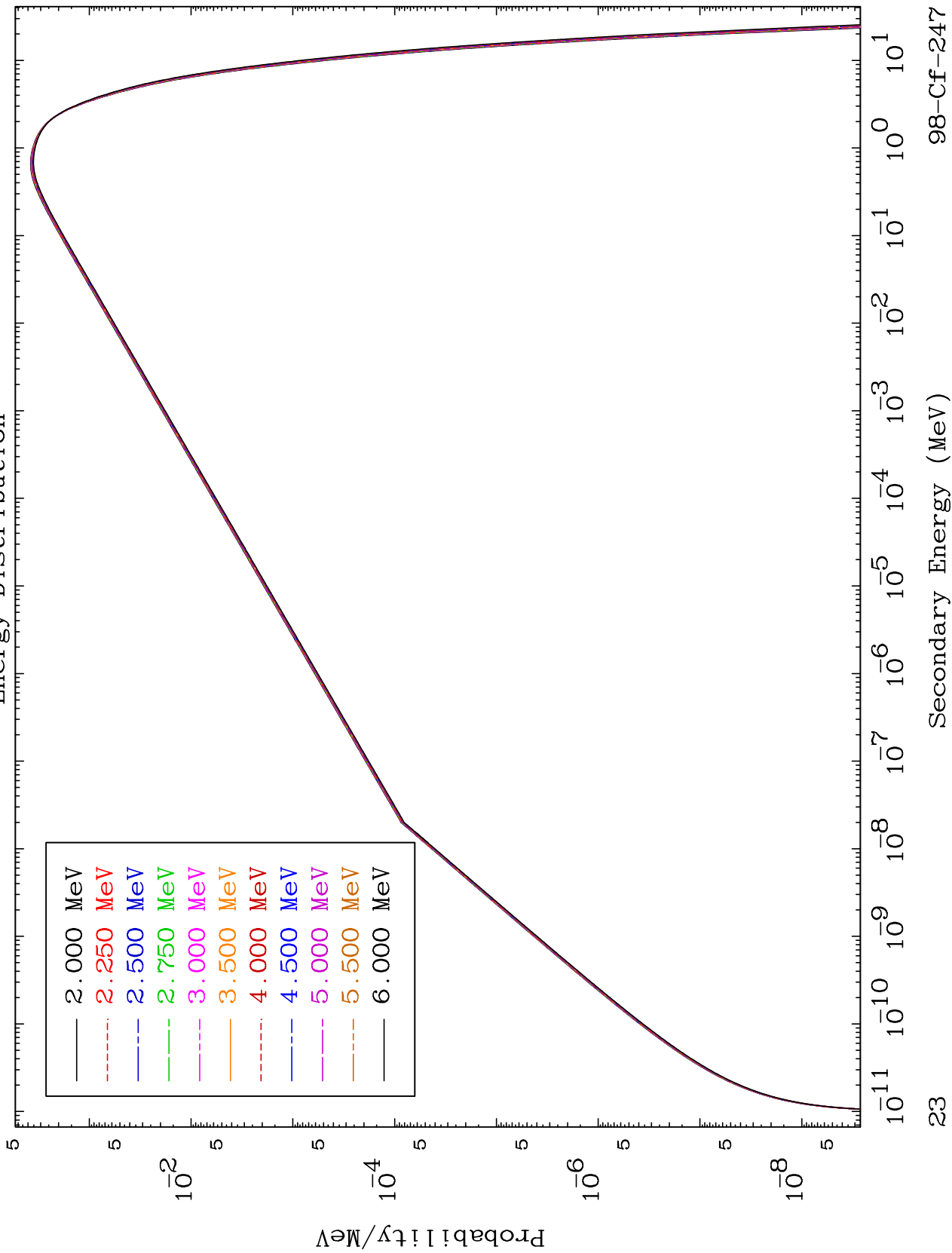
Secondary Energy (MeV)

22

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### Fission Energy Distribution

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98-Cf-247

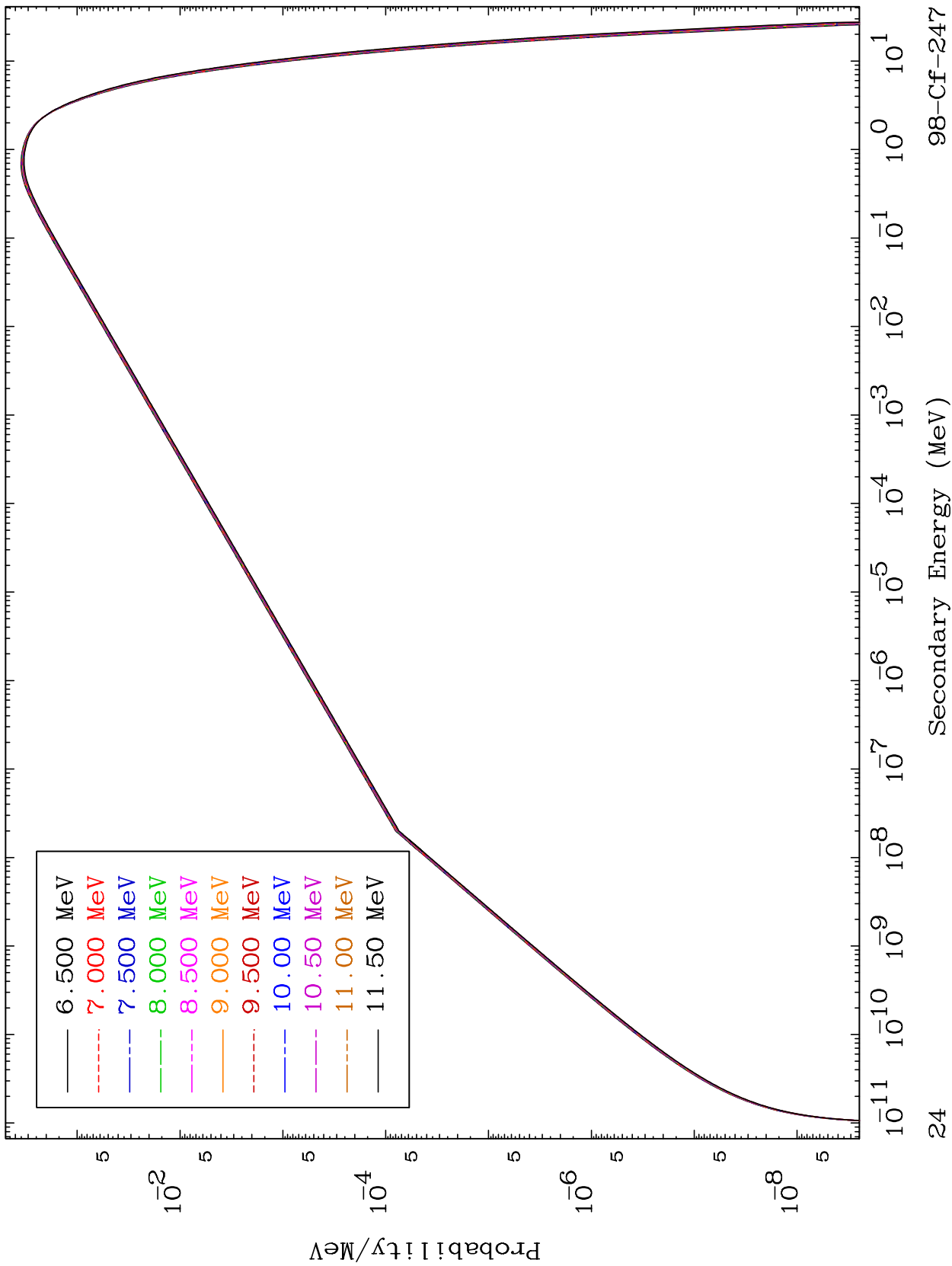
Secondary Energy (MeV)



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Fission  
Energy Distribution

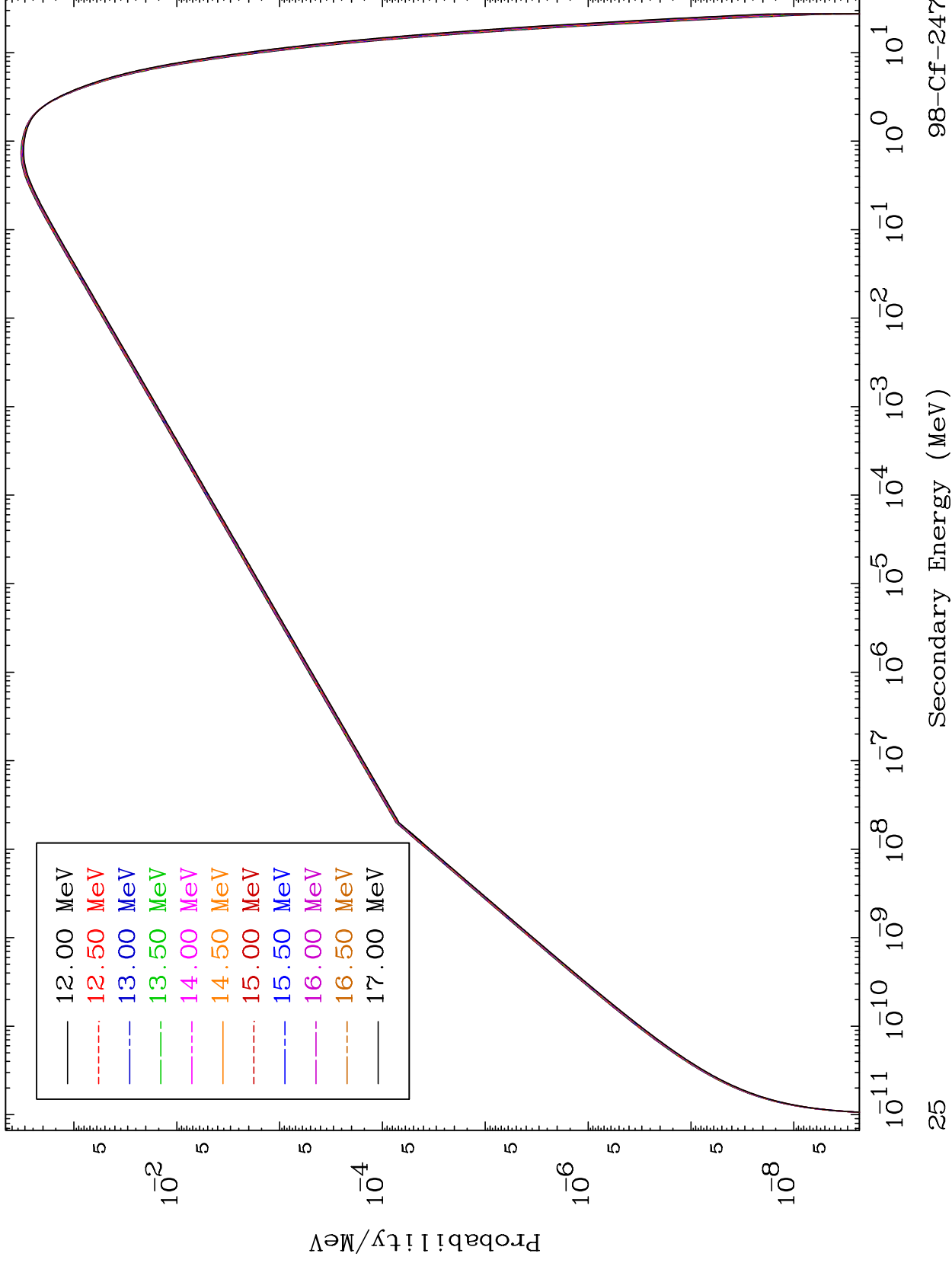
98-Cf-247



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### Fission Energy Distribution

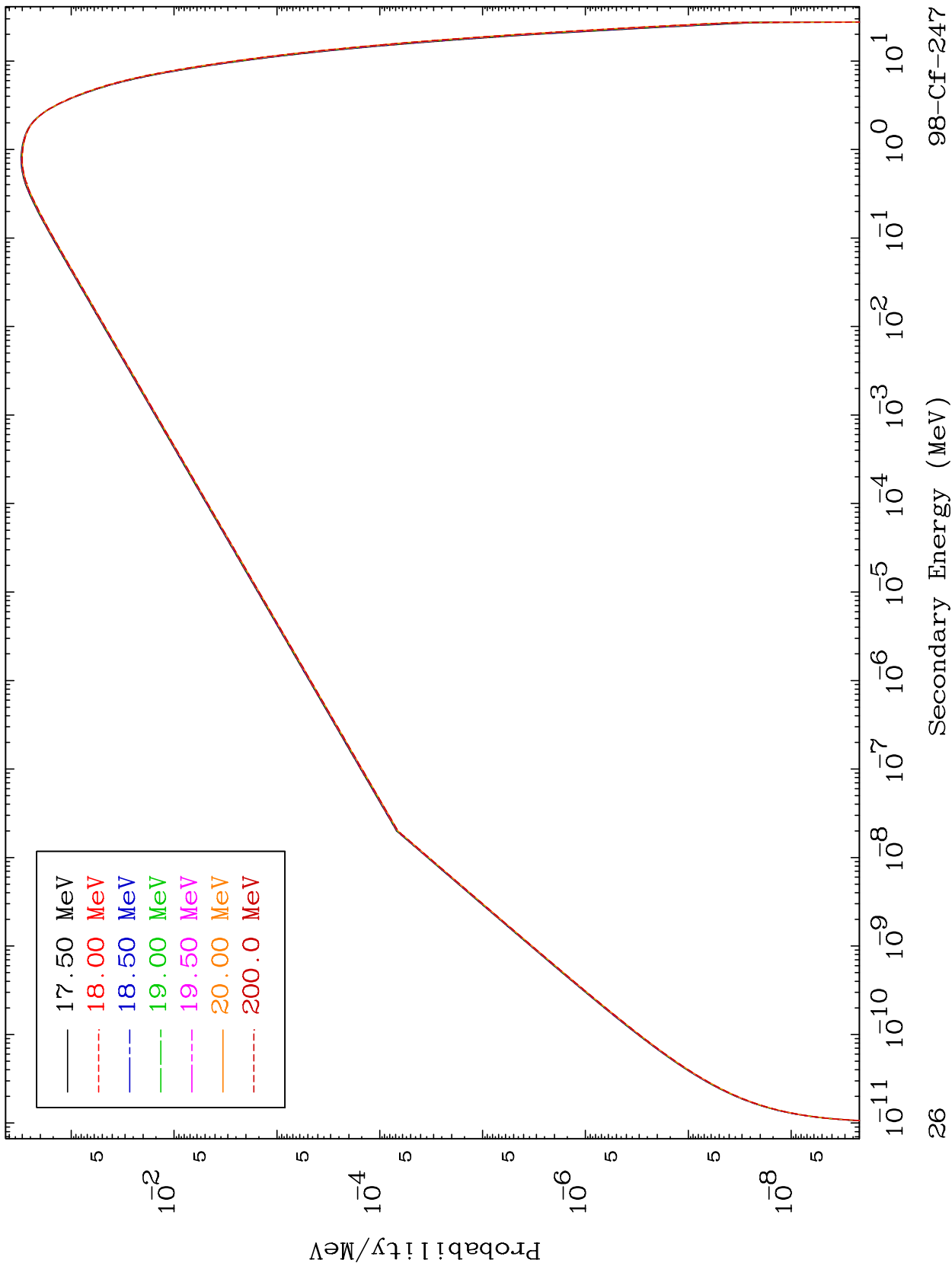
98-Cf-247



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### Fission Energy Distribution

98-Cf-247

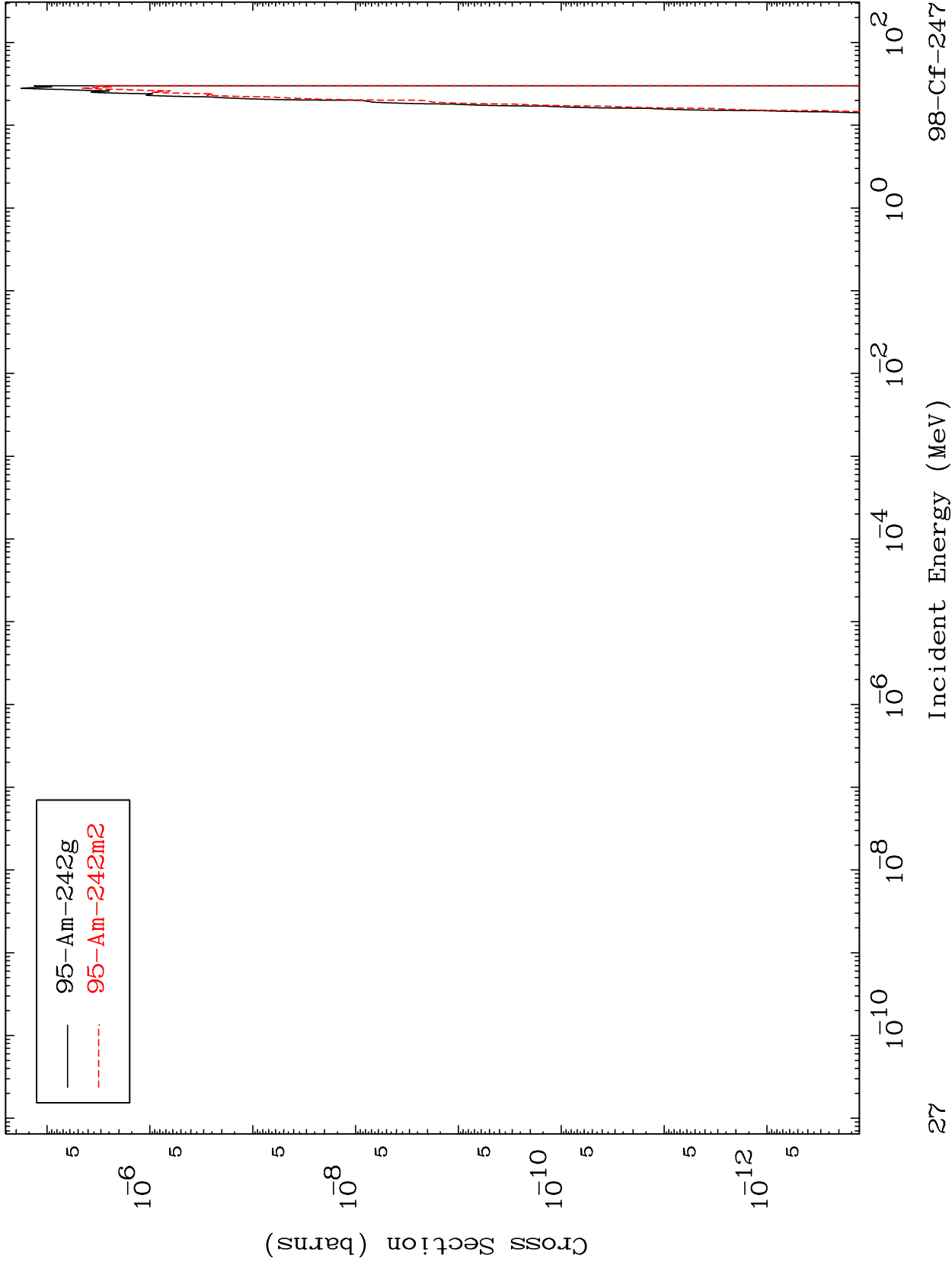


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(n,n') p  $\alpha$

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Radionuclide Production Cross Section



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(n,d)  $\alpha$

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Radionuclide Production Cross Section

