

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

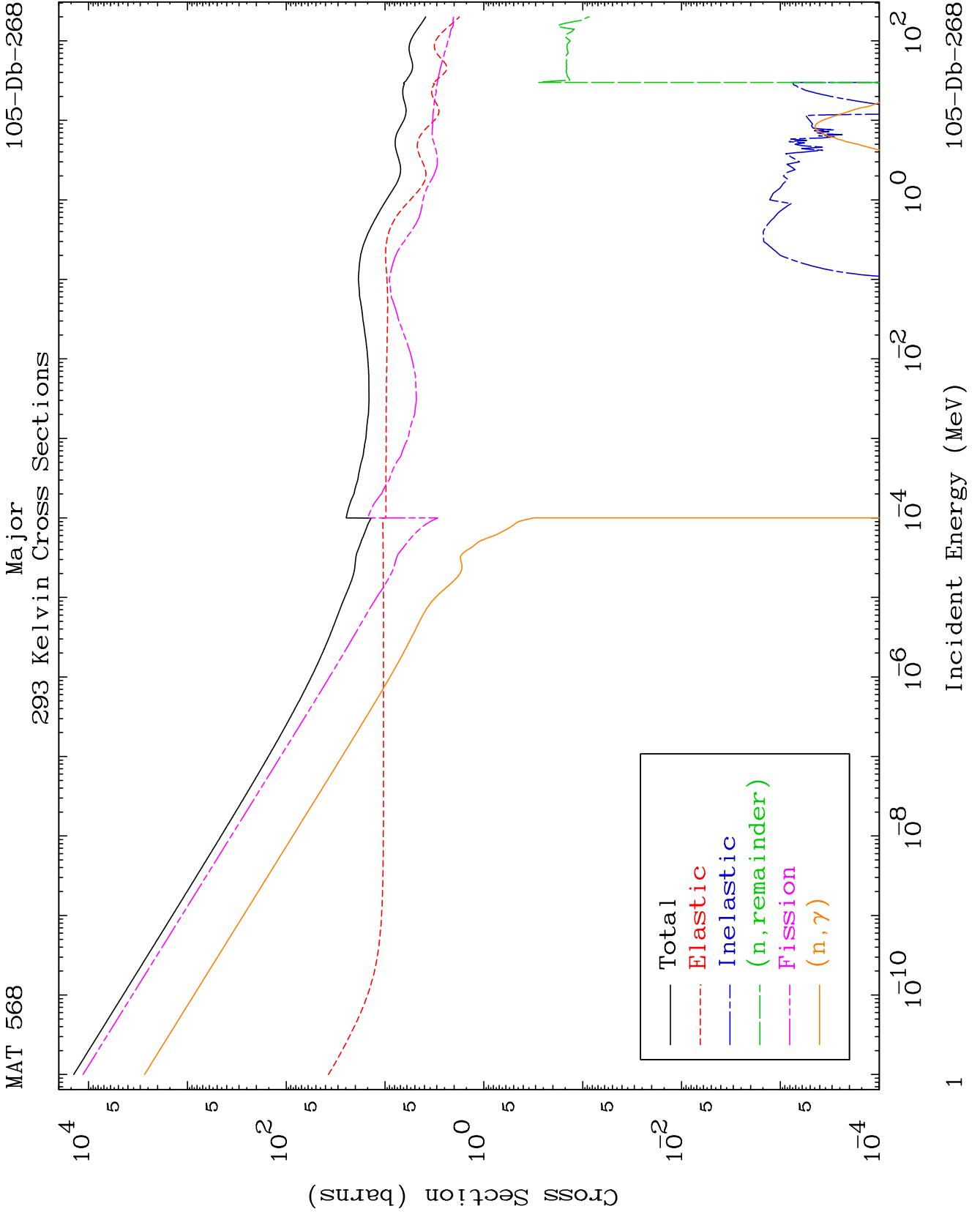
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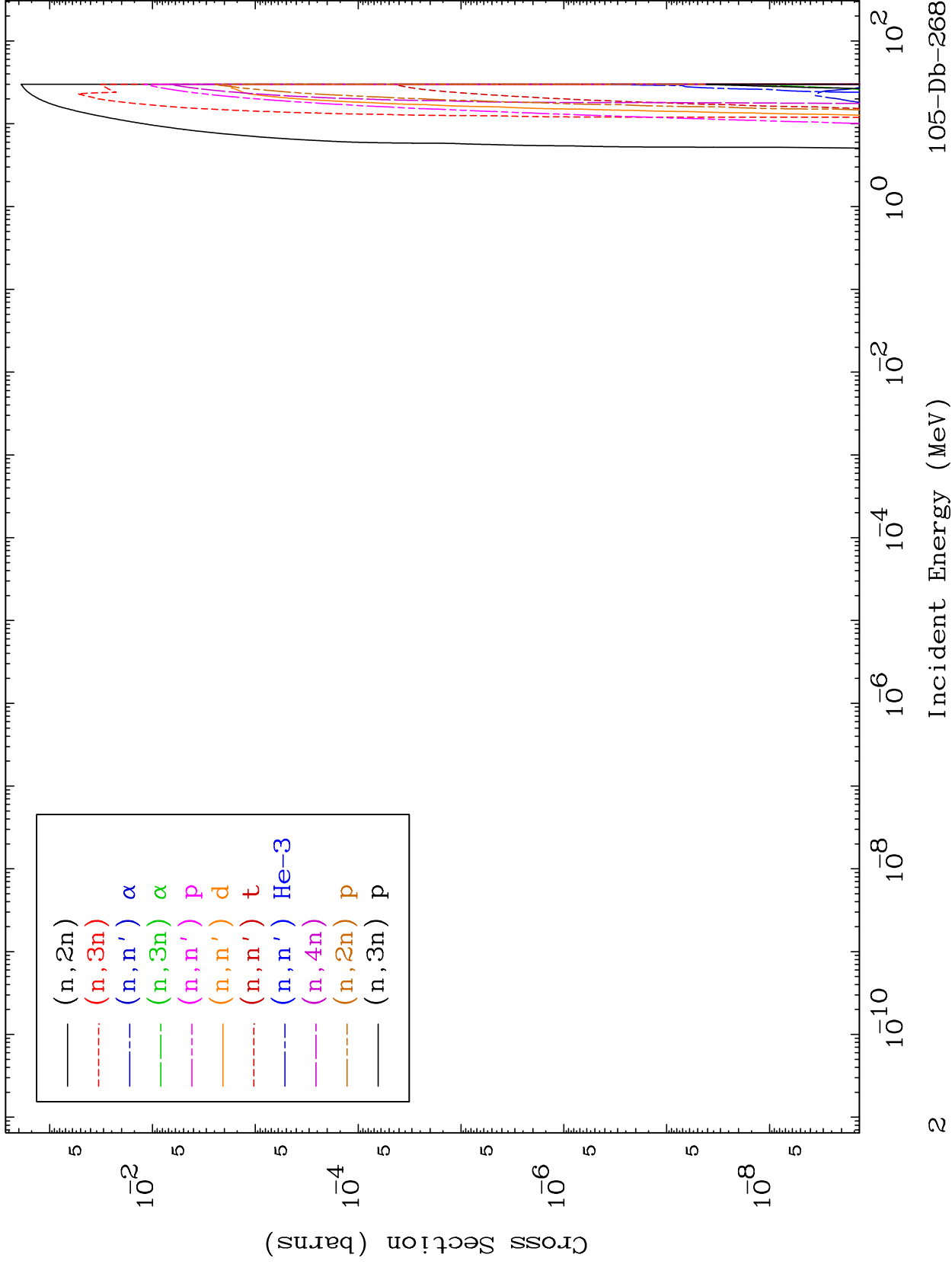
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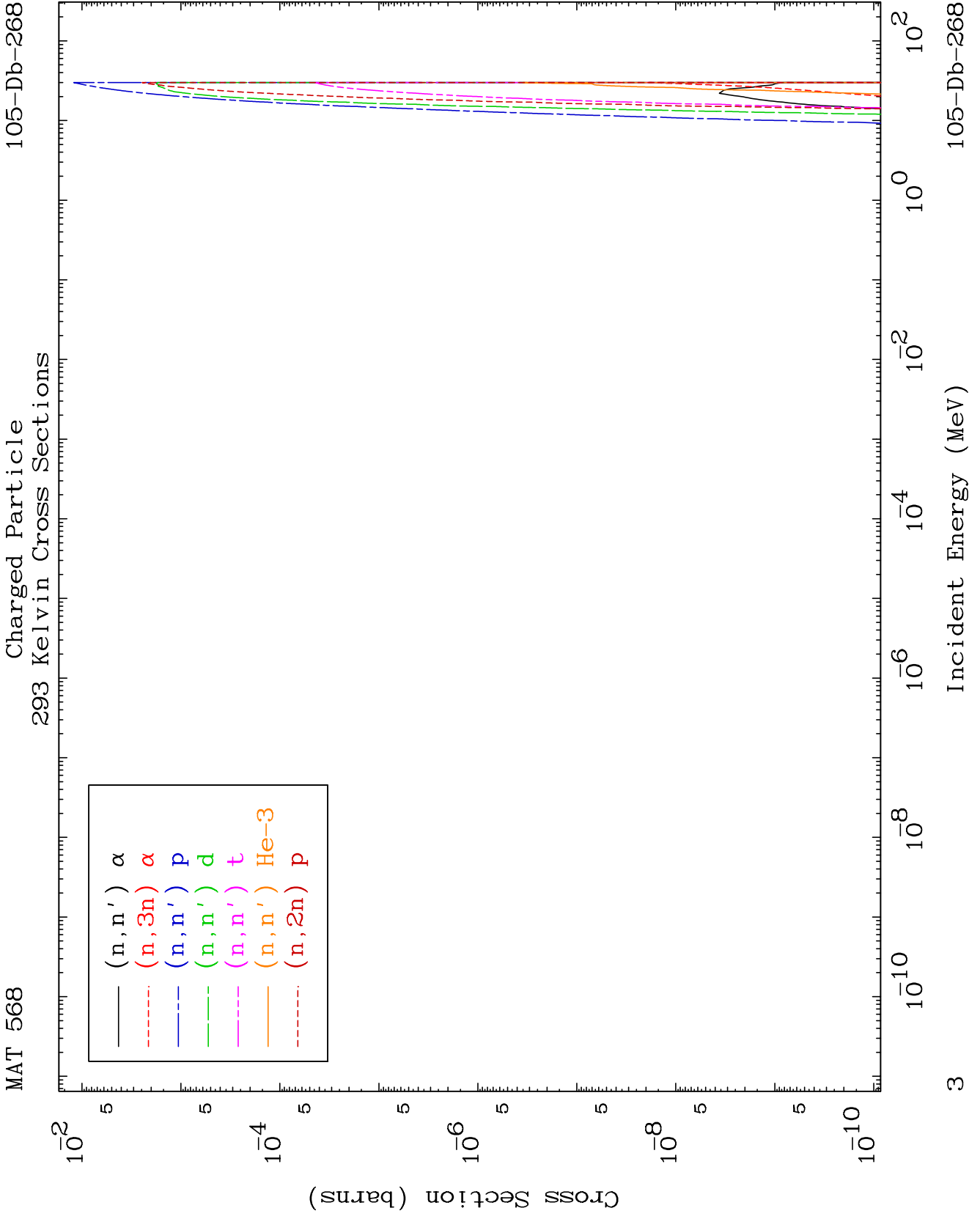


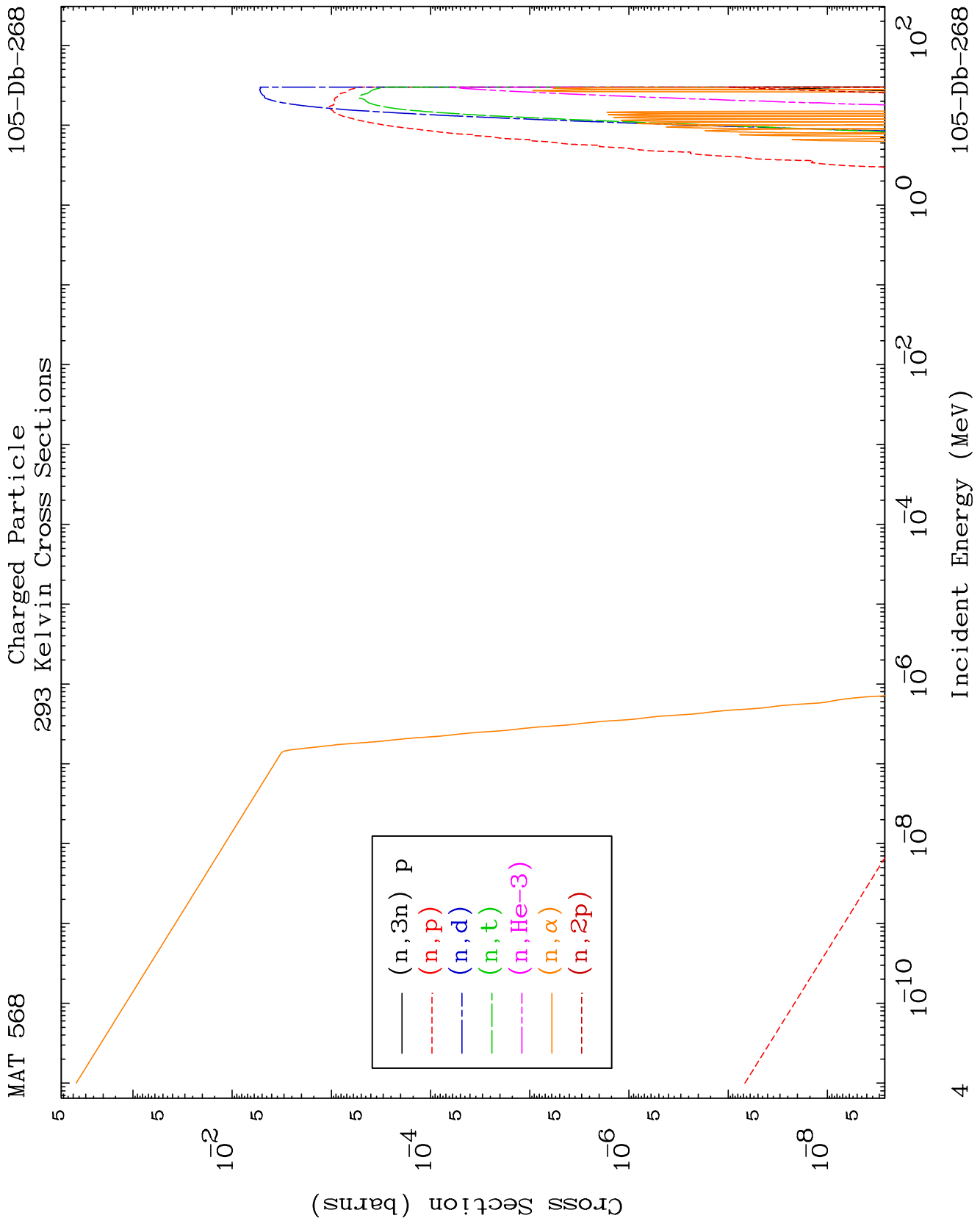
MAT 568

Neutron Production
293 Kelvin Cross Sections

105-Db-268



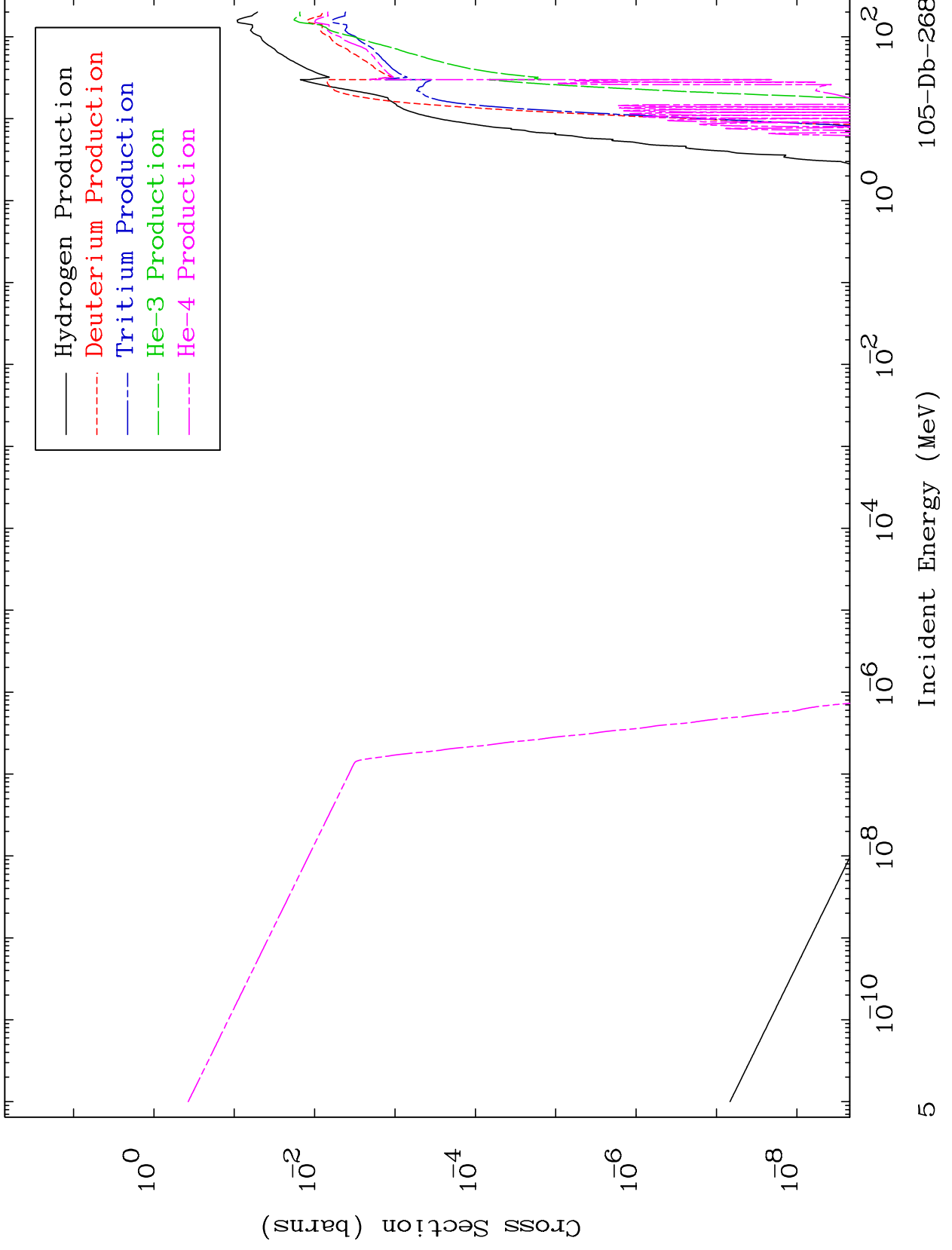


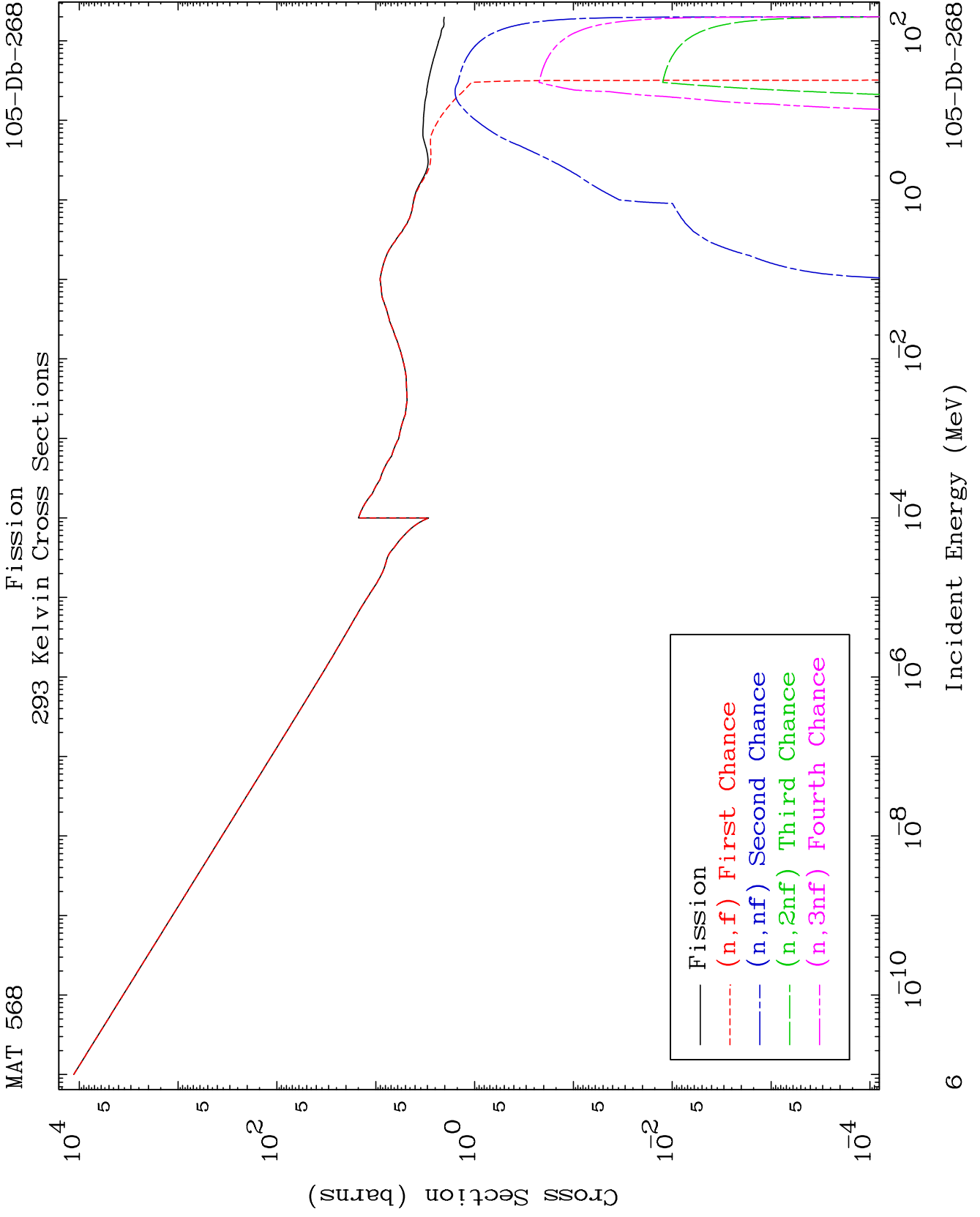


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Particle Production
293 Kelvin Cross Sections

105-Db-268

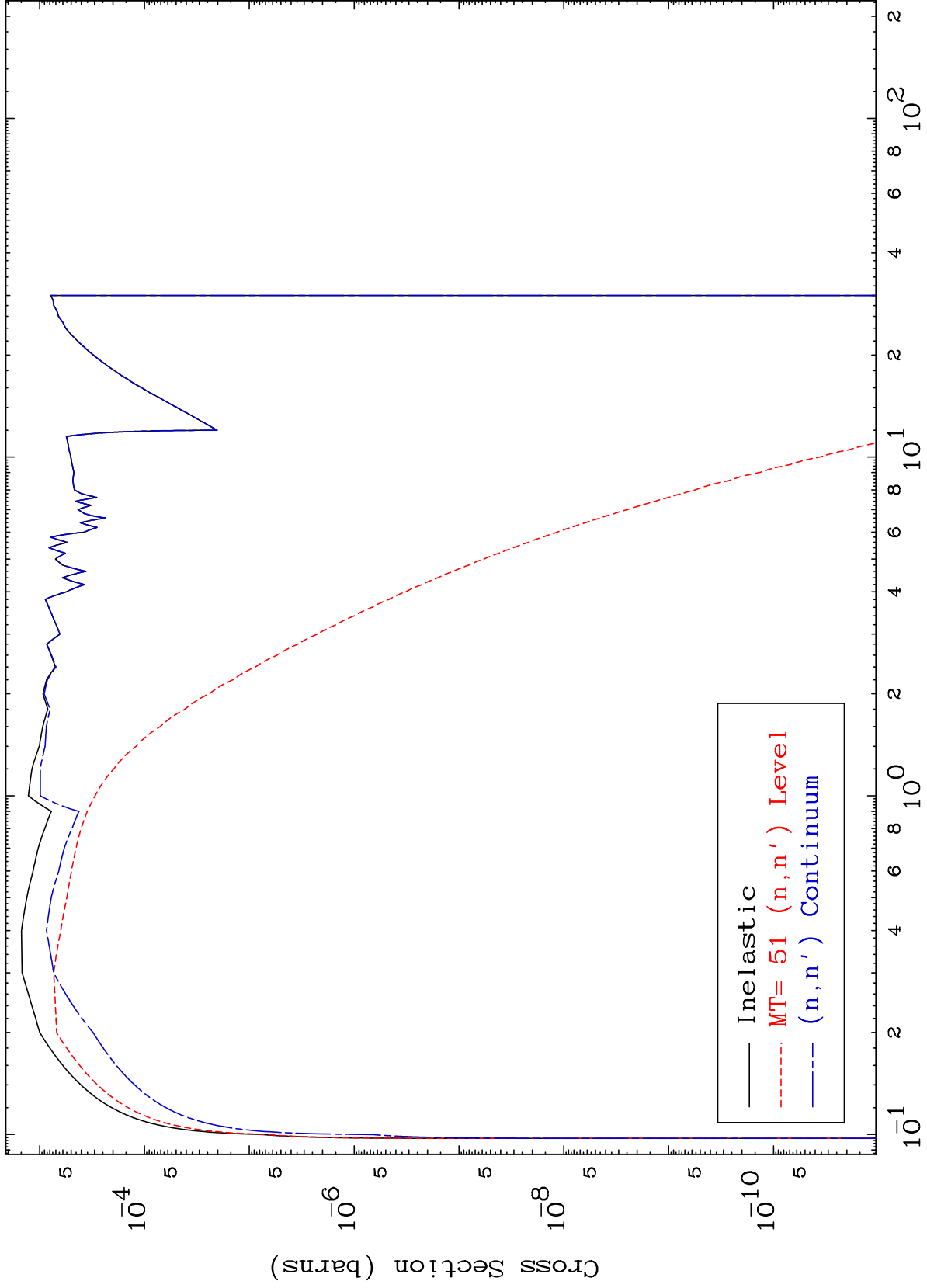




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(n,n') Level
293 Kelvin Cross Sections

105-Db-268



Incident Energy (MeV)

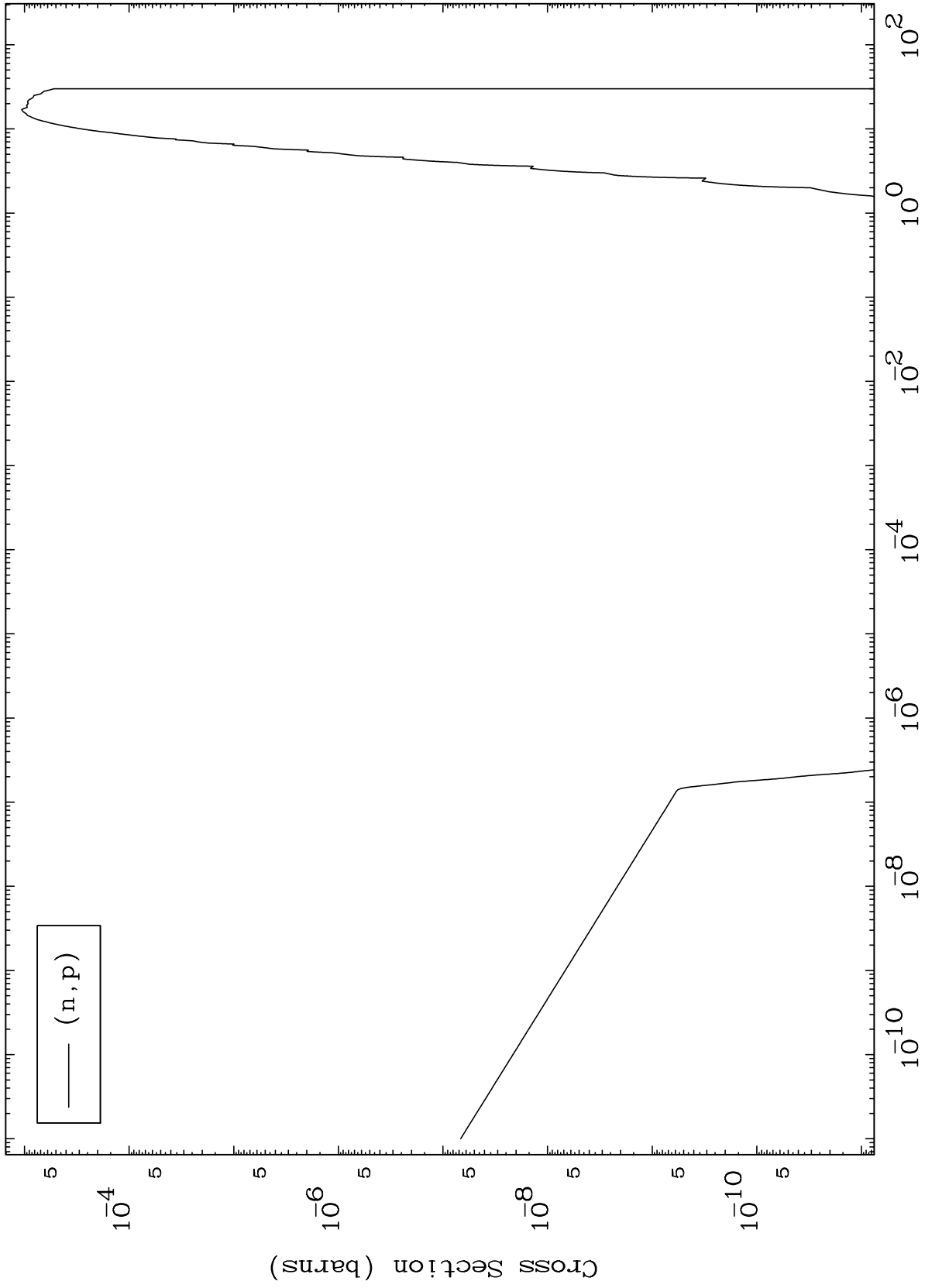
105-Db-268

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MAT 568

(n,p) Levels
293 Kelvin Cross Sections

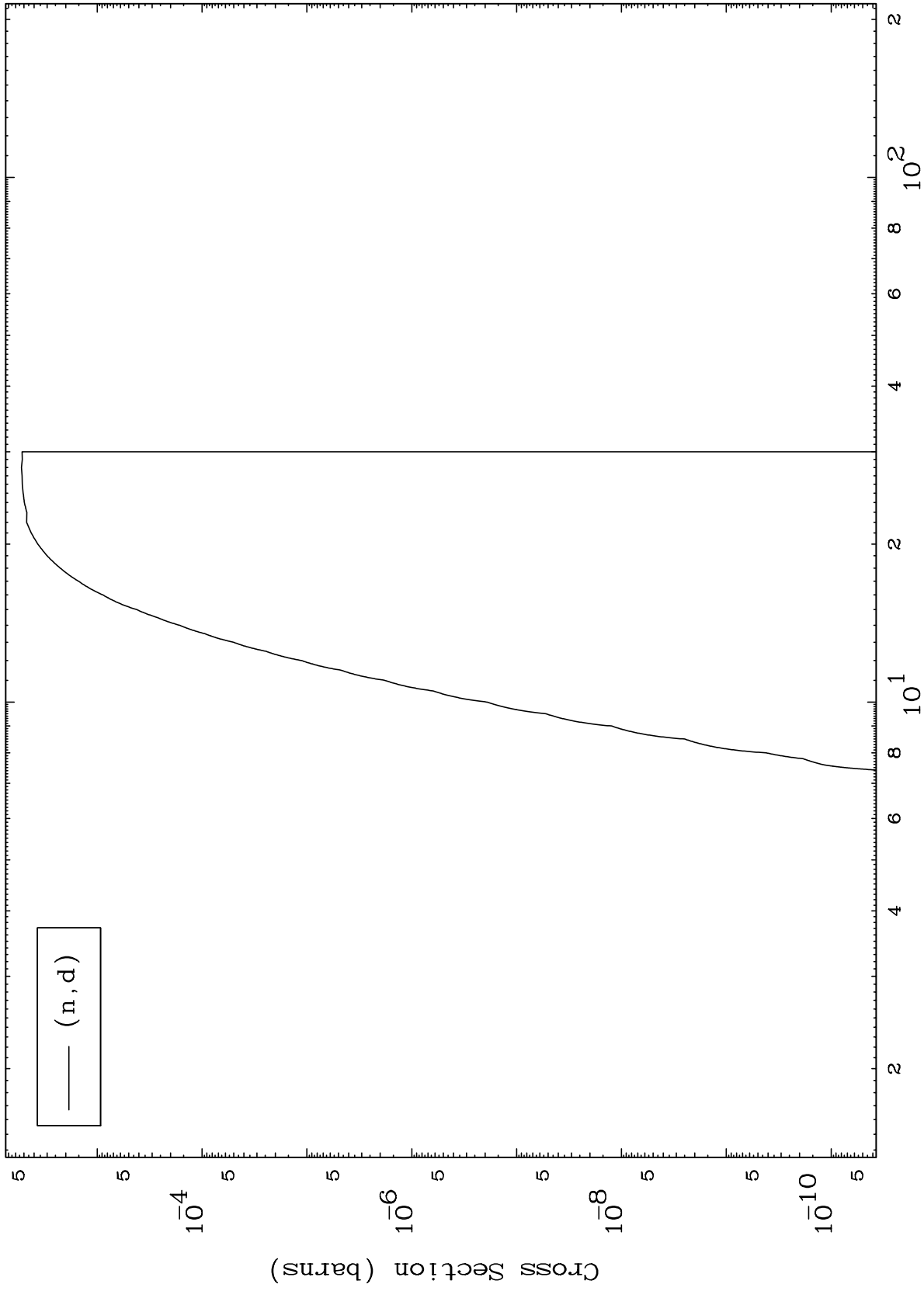
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(n,d) Levels
293 Kelvin Cross Sections

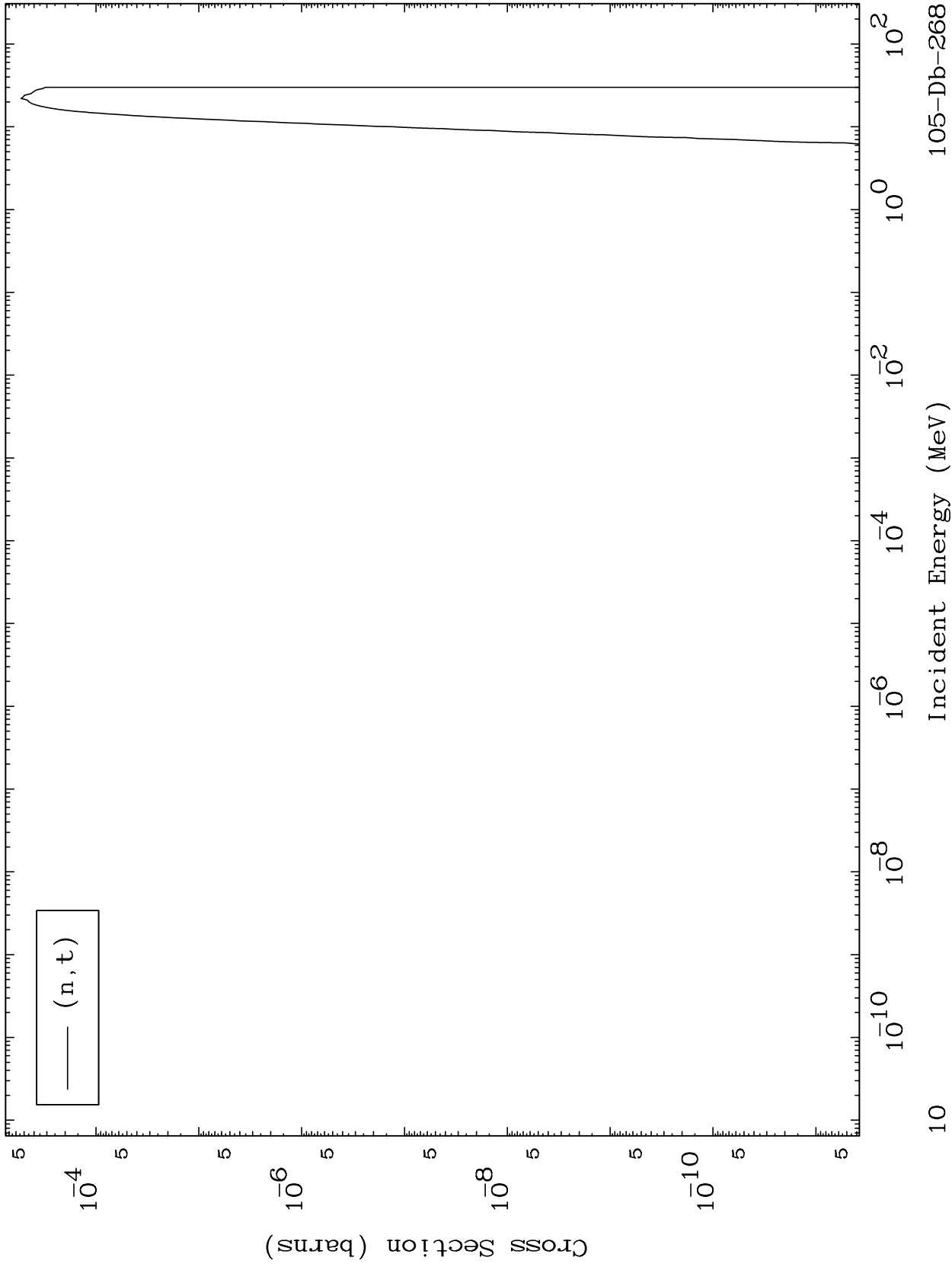
105-Db-268



MAT 568

(n,t) Levels
293 Kelvin Cross Sections

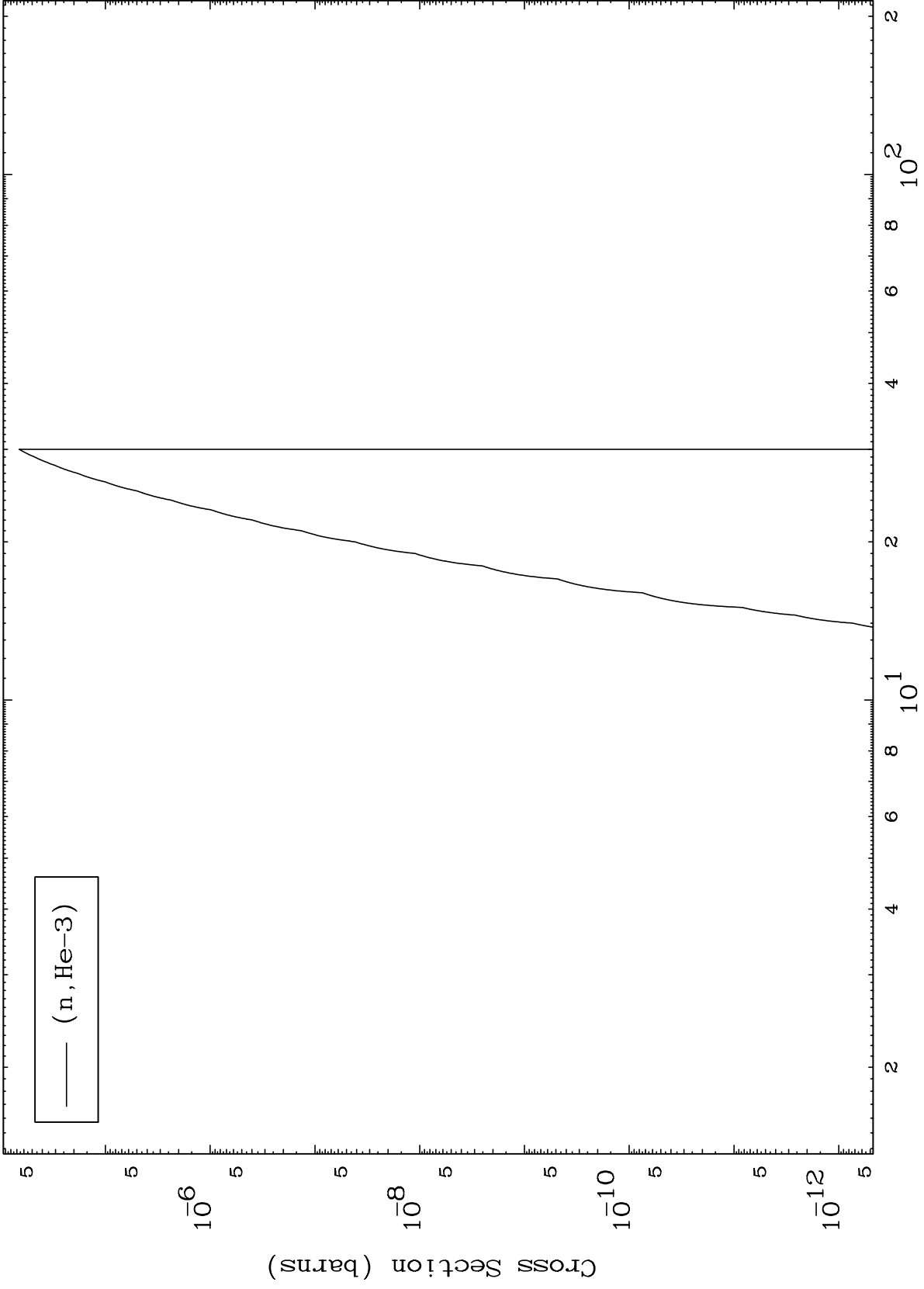
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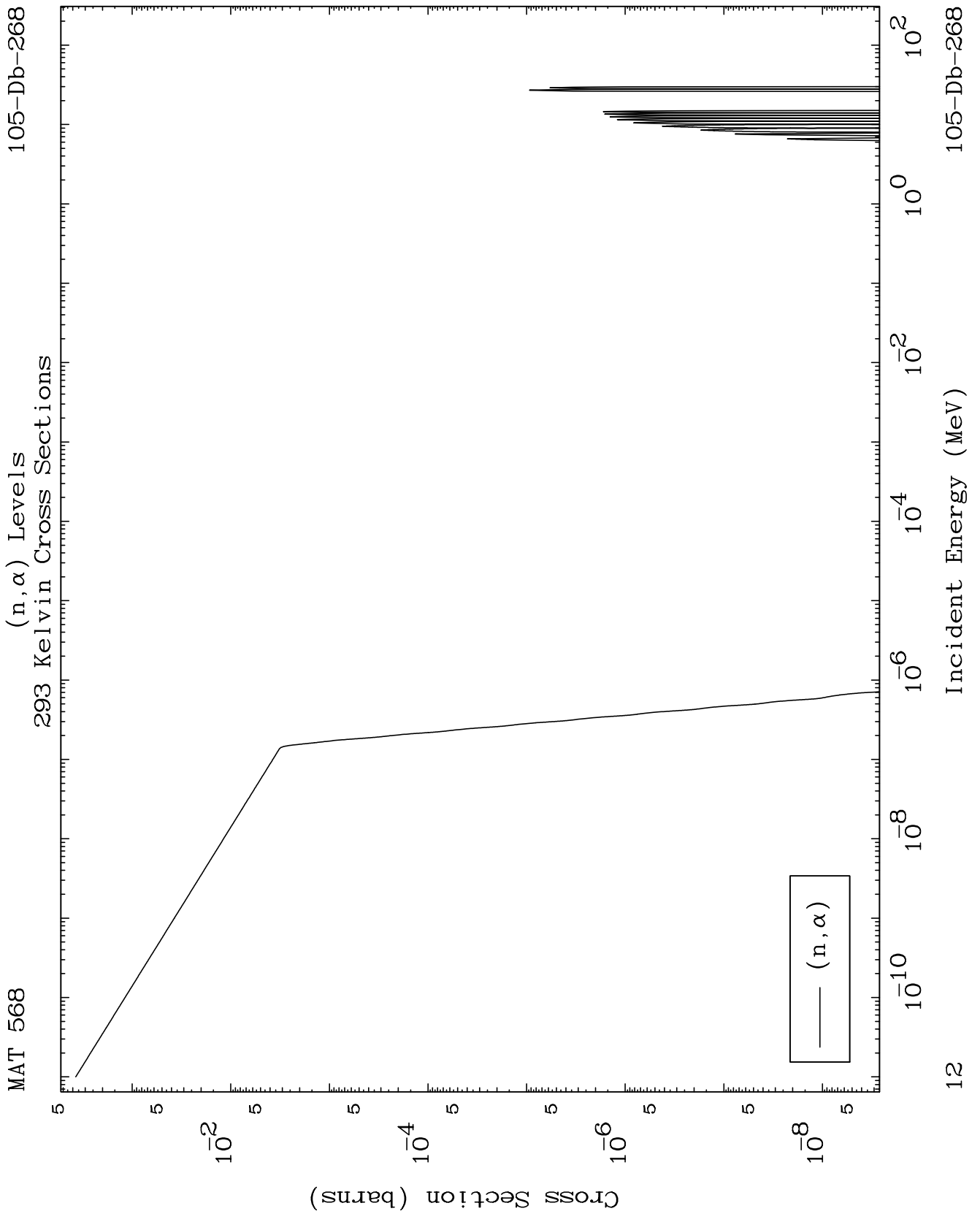


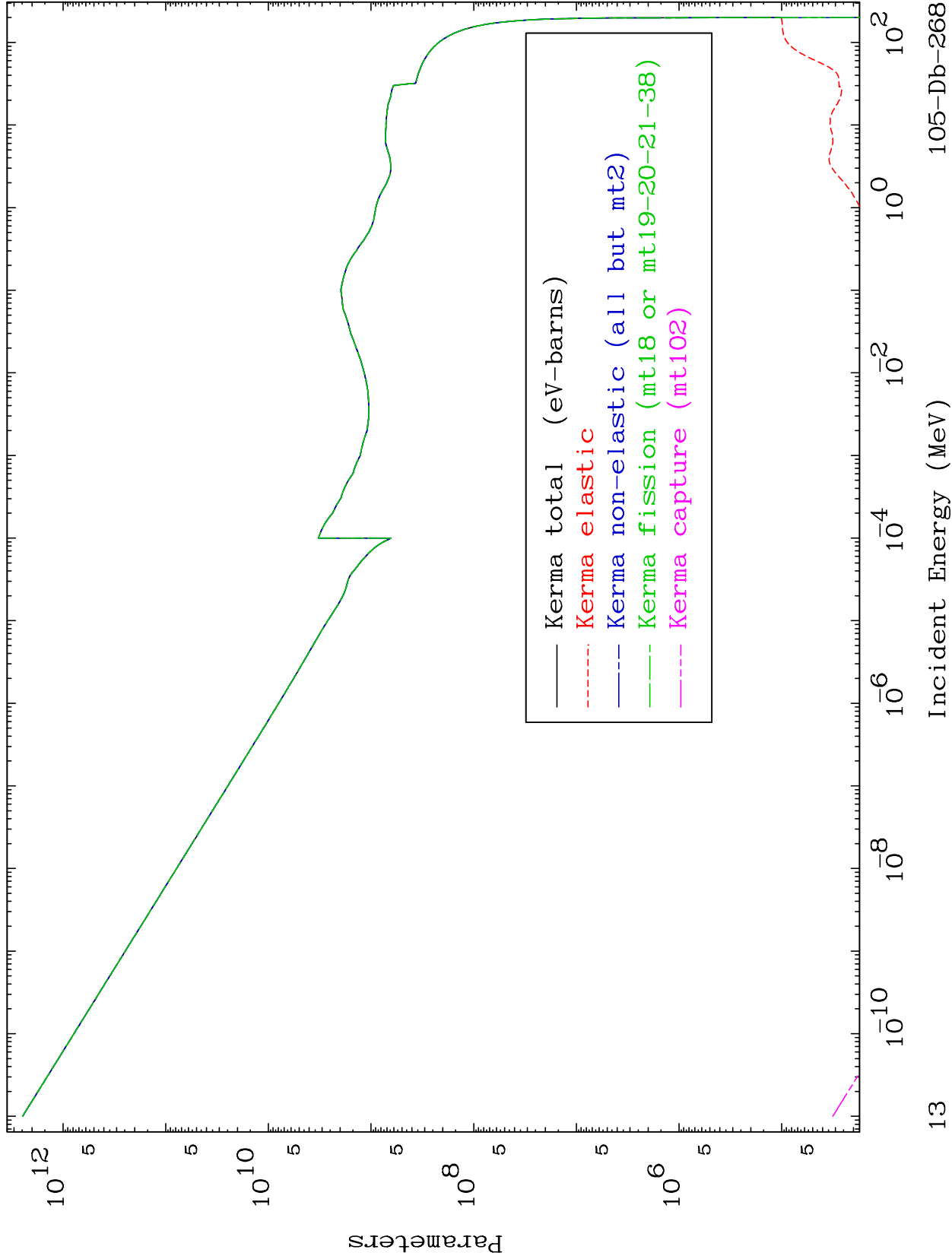
MAT 568

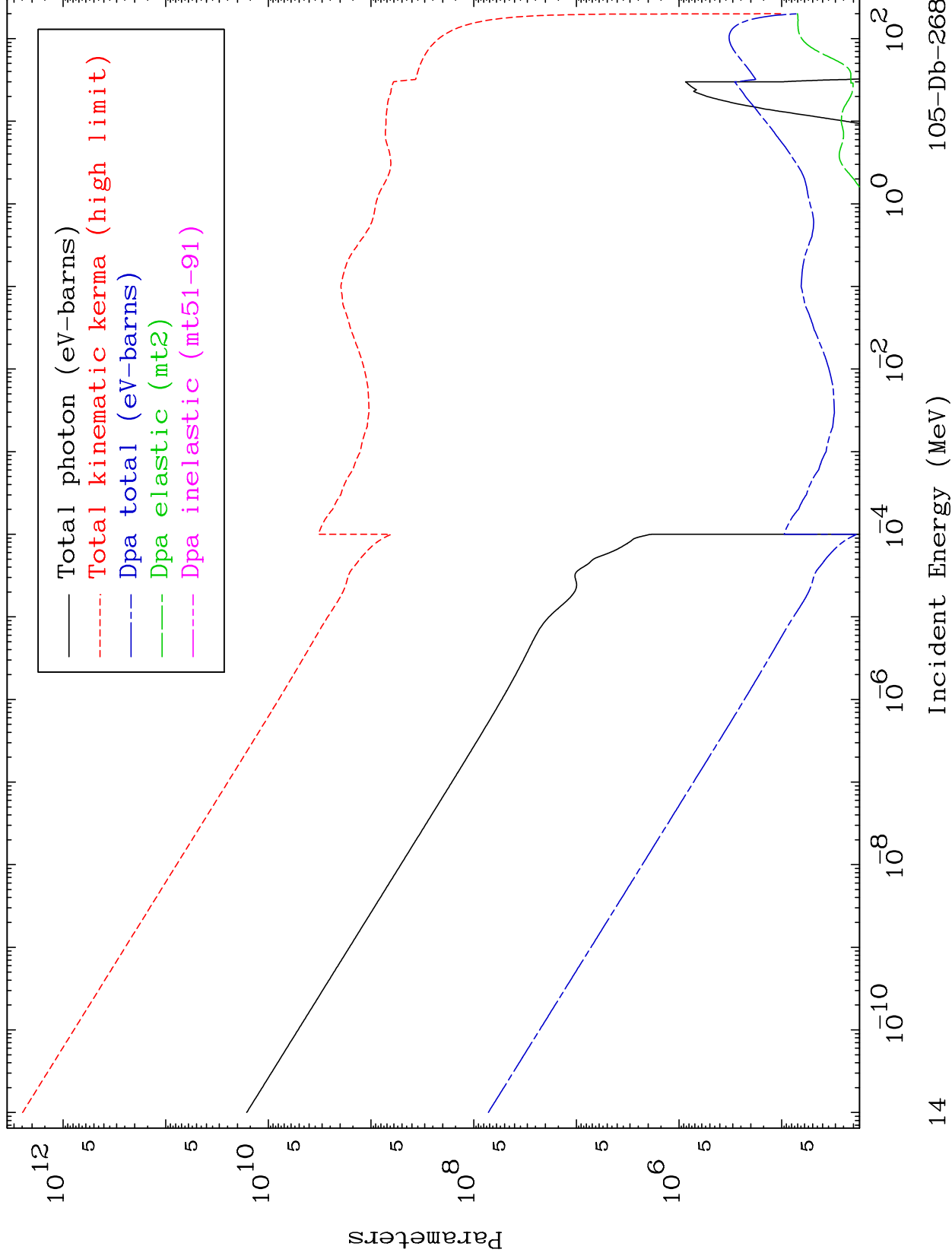
(n,He3) Levels
293 Kelvin Cross Sections

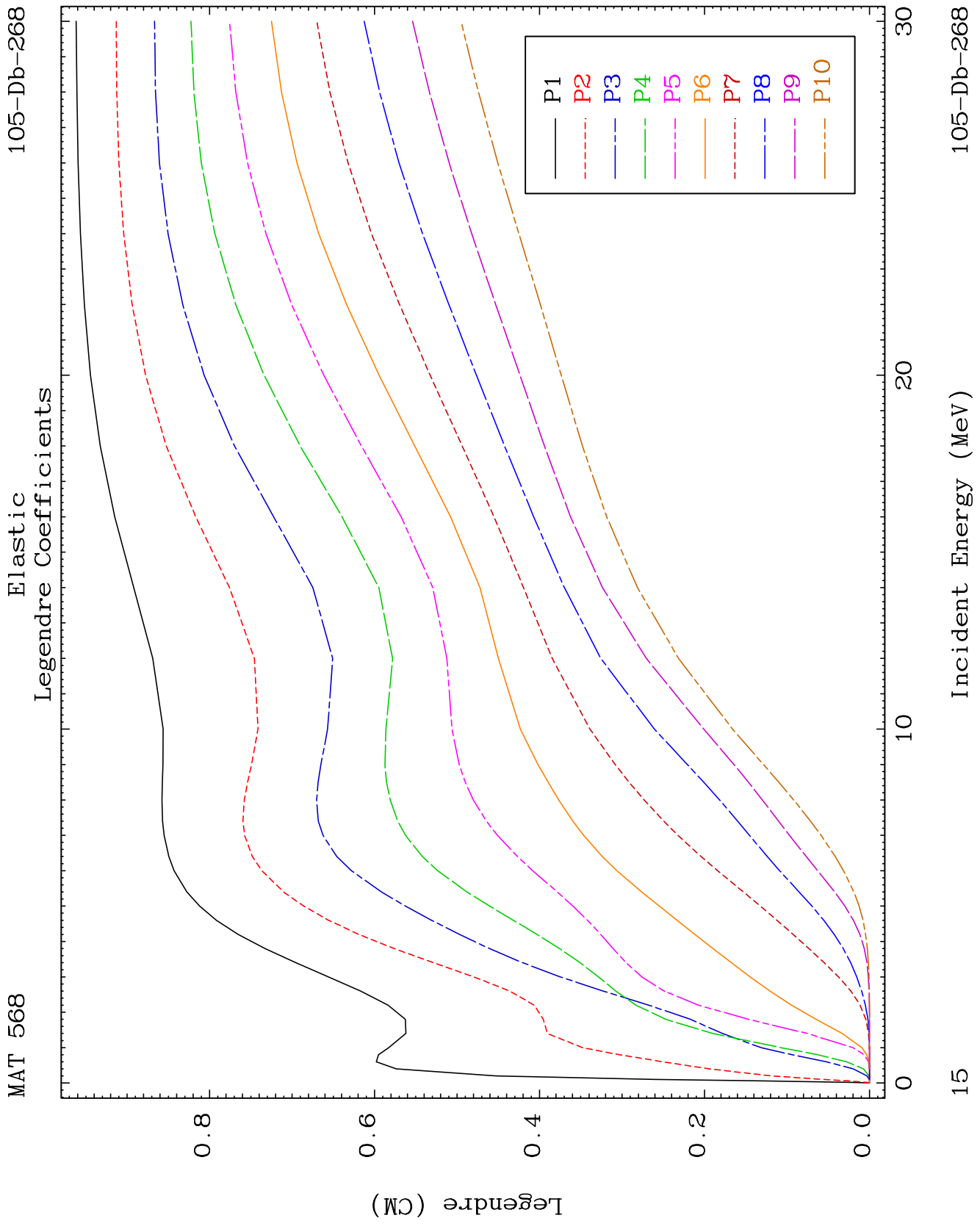
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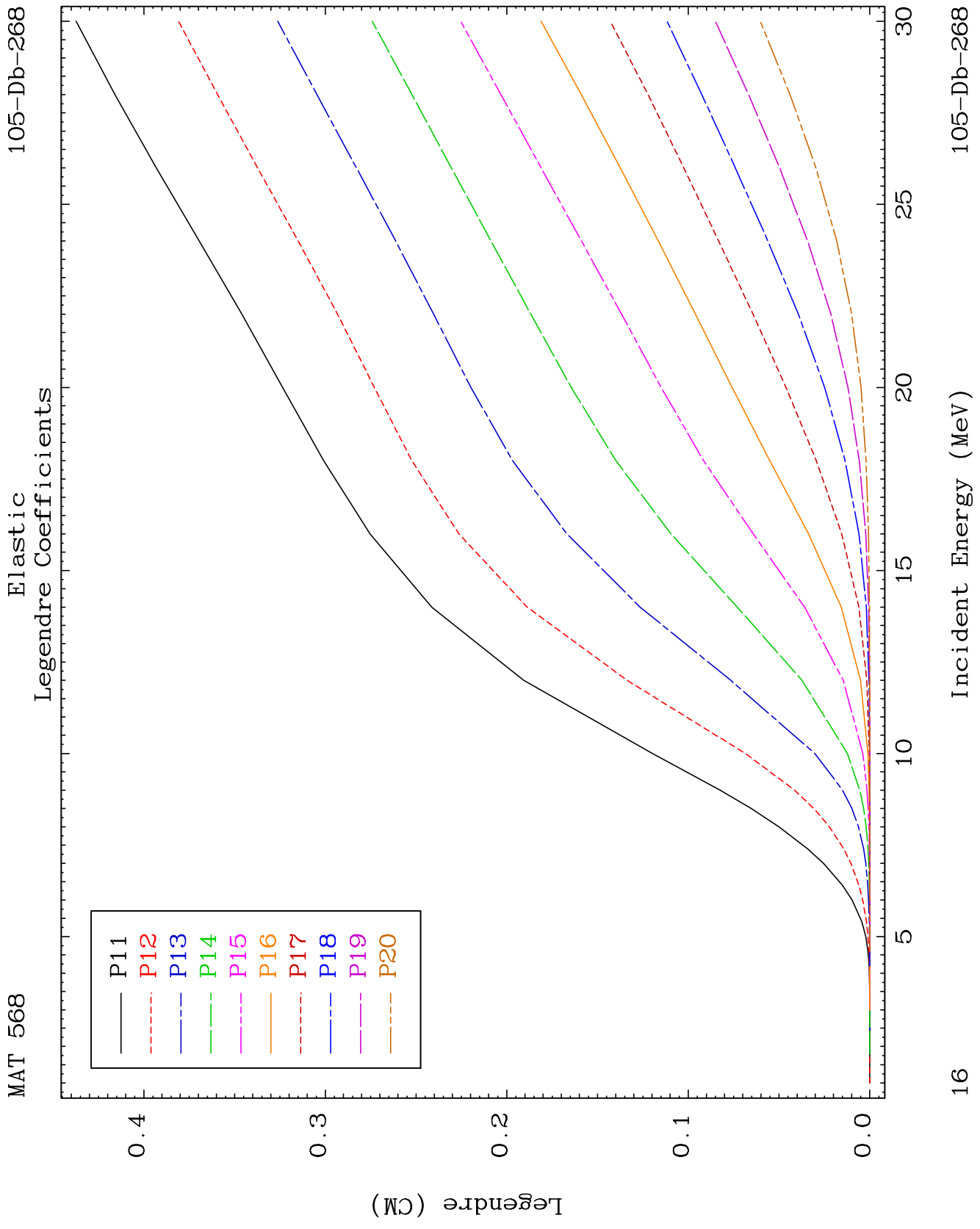


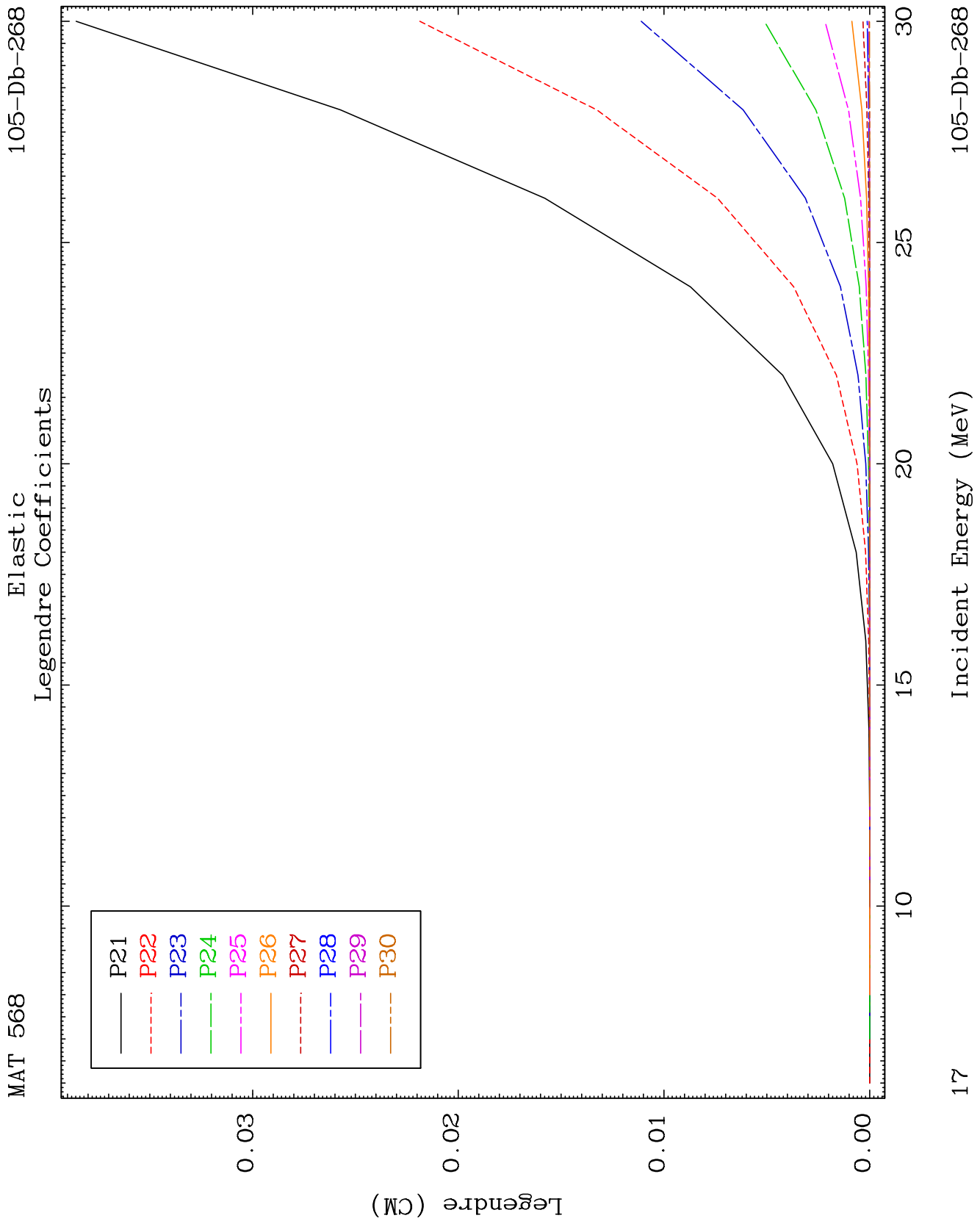












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Elastic Legendre Coefficients

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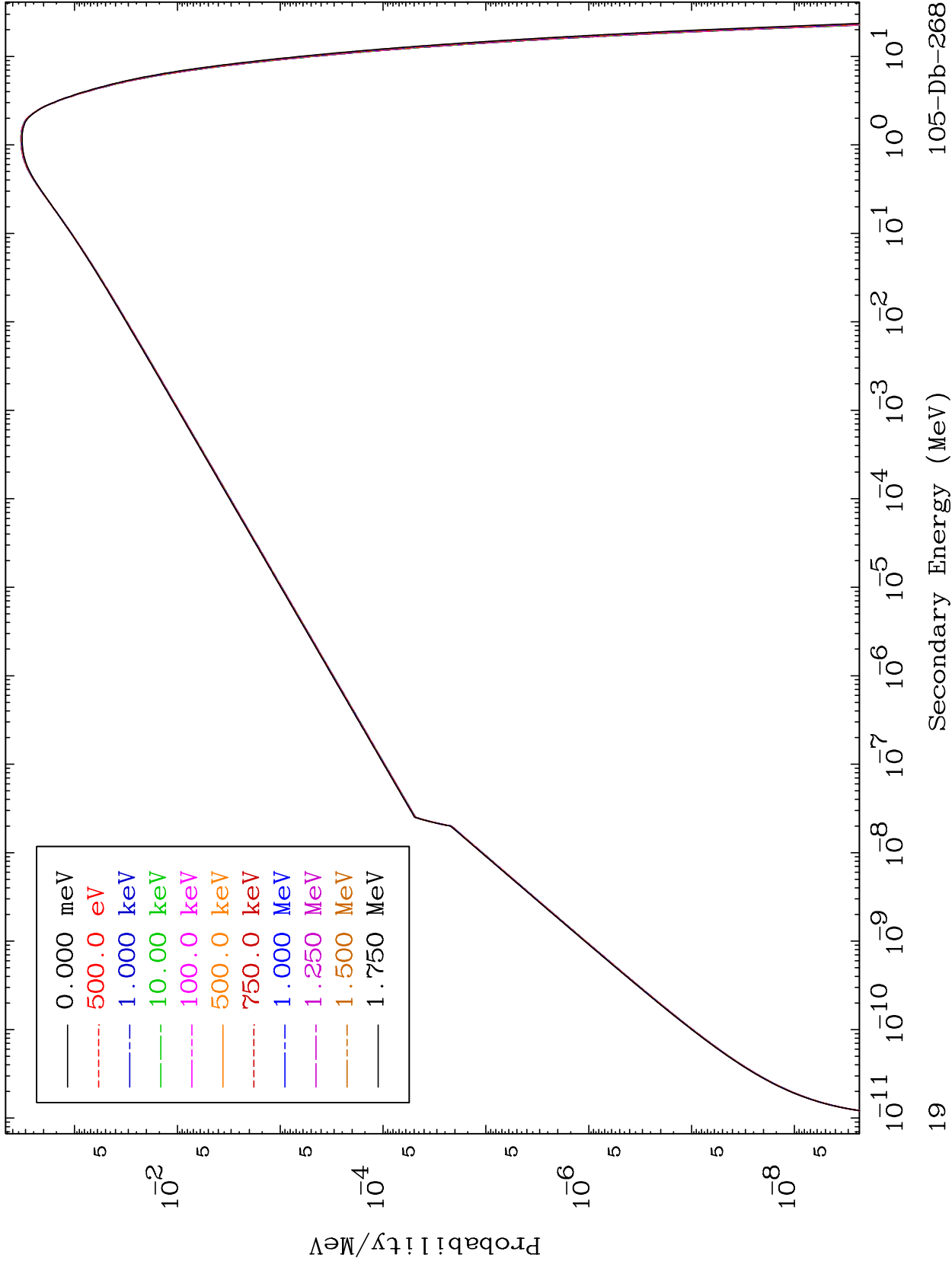
Incident Energy (MeV)

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Fission Energy Distribution

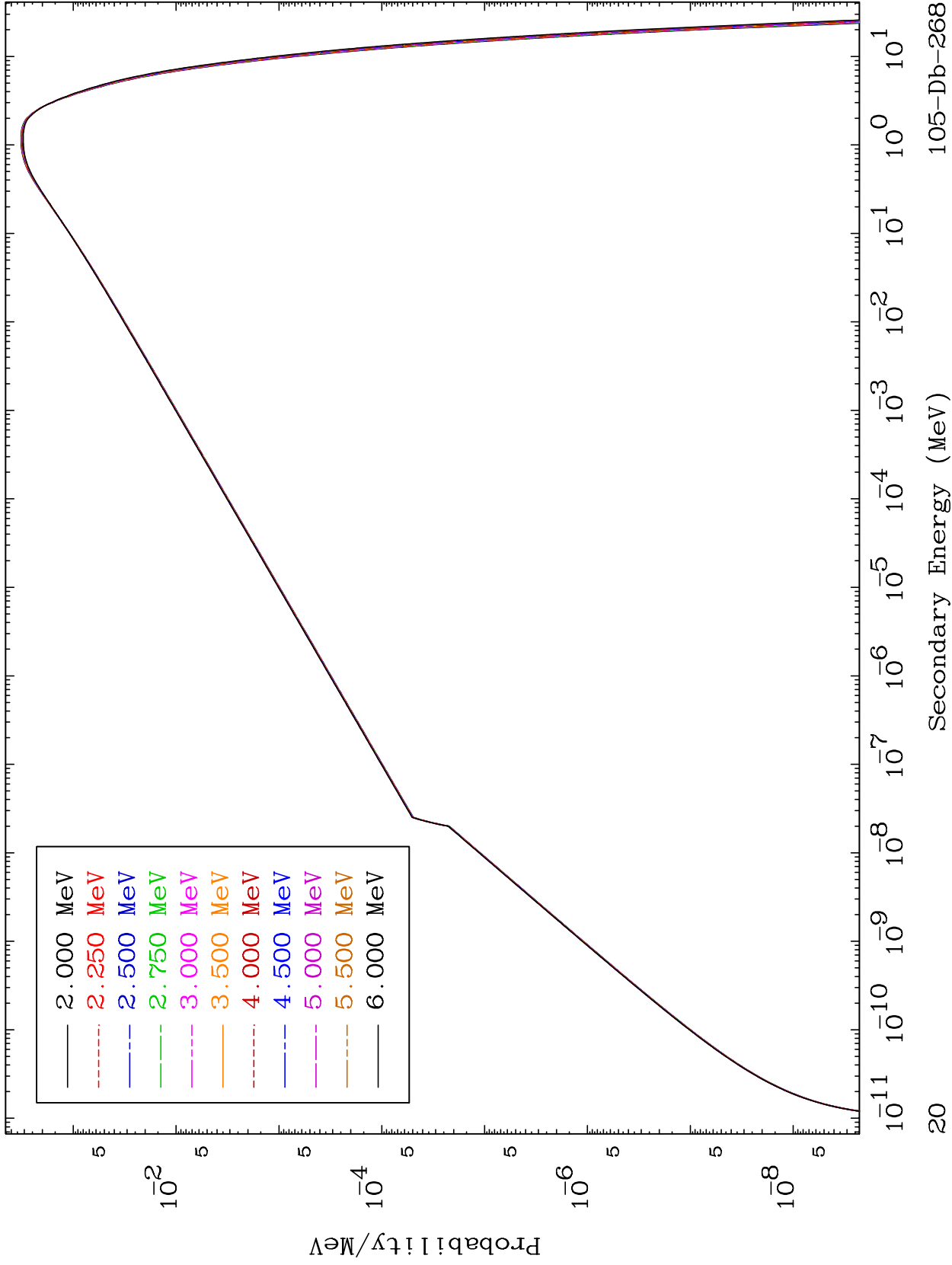
105-Db-268



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Fission Energy Distribution

105-Db-268



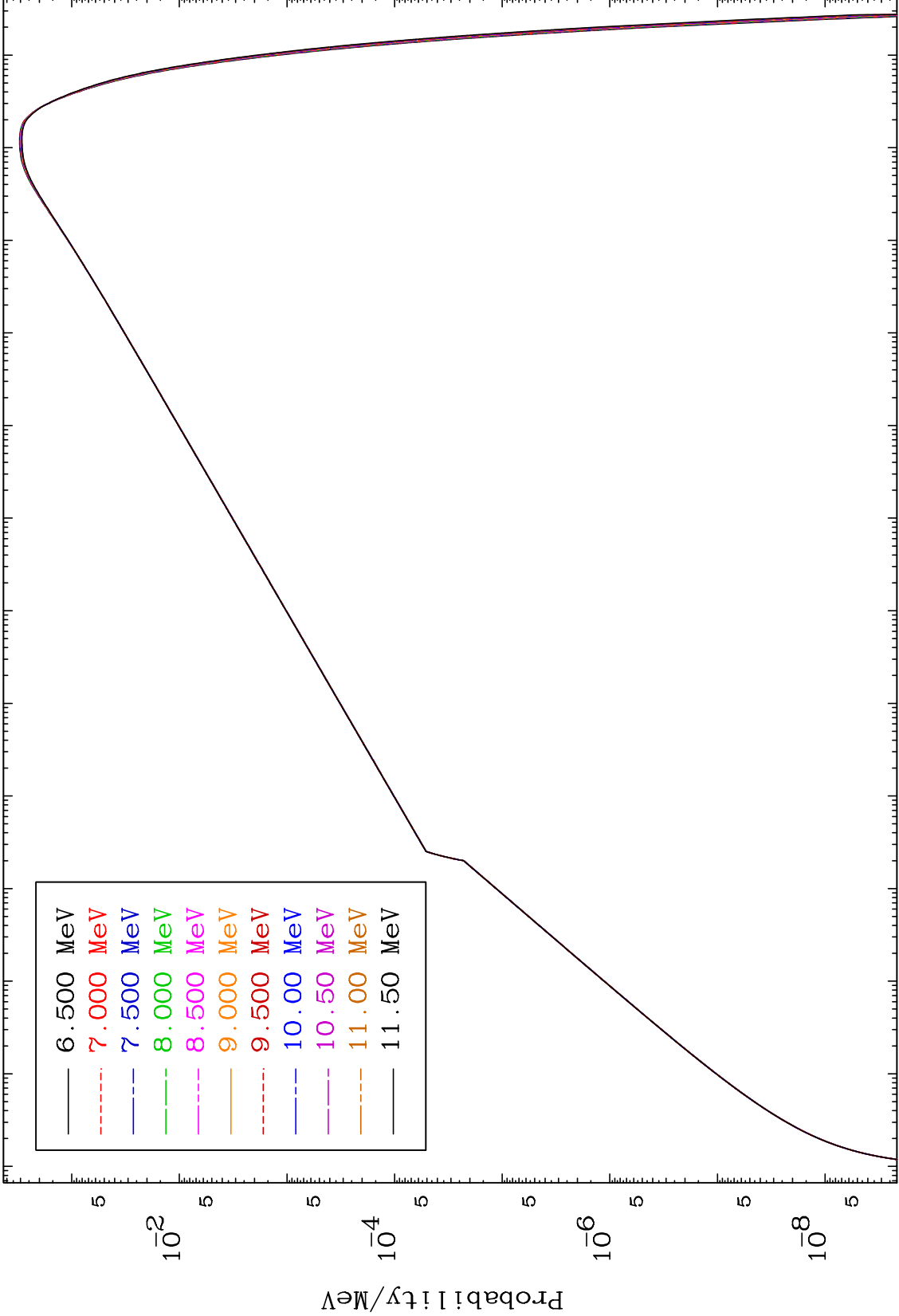
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Fission

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Energy Distribution



10⁻¹¹ 10⁻¹⁰ 10⁻⁹ 10⁻⁸ 10⁻⁷ 10⁻⁶ 10⁻⁵ 10⁻⁴ 10⁻³ 10⁻² 10⁻¹ 10⁰ 10¹

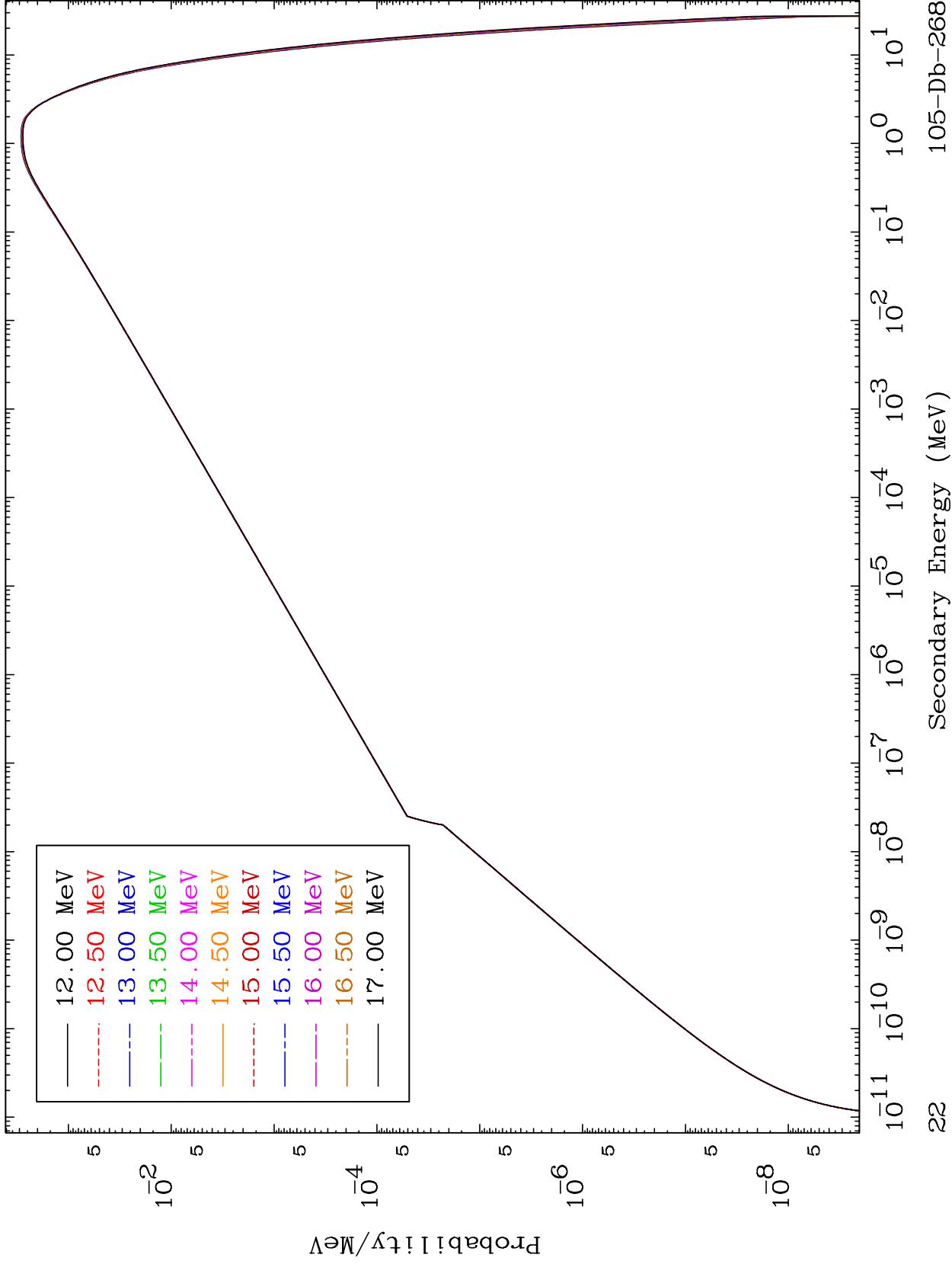
Secondary Energy (MeV)

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Fission Energy Distribution

105-Db-268



MAT 568

Fission Energy Distribution

105-Db-268

