

Program EVALPLOT  
(Version 2018-1)

by

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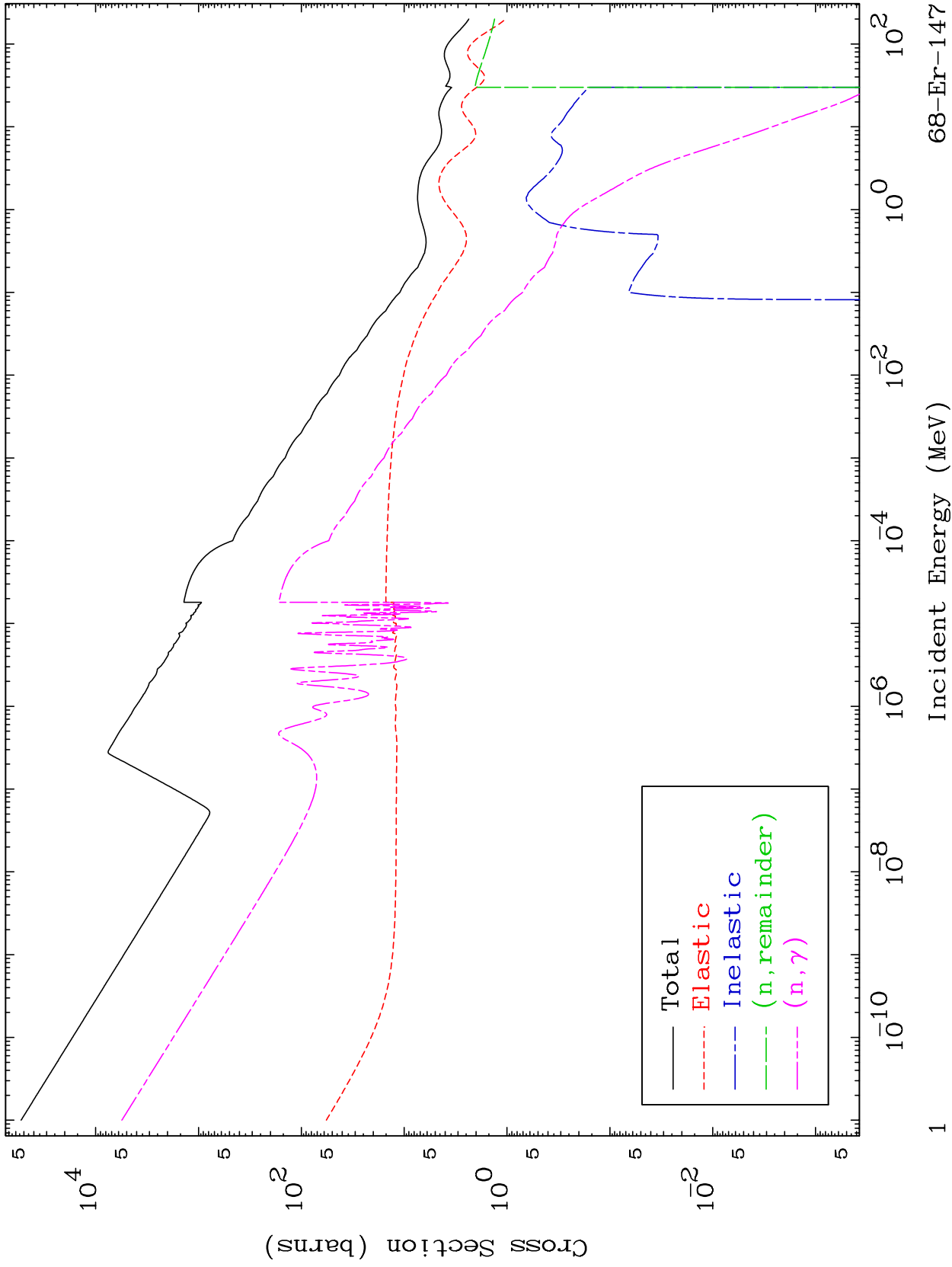
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6780

Major  
293 Kelvin Cross Sections

68-Er-147



1

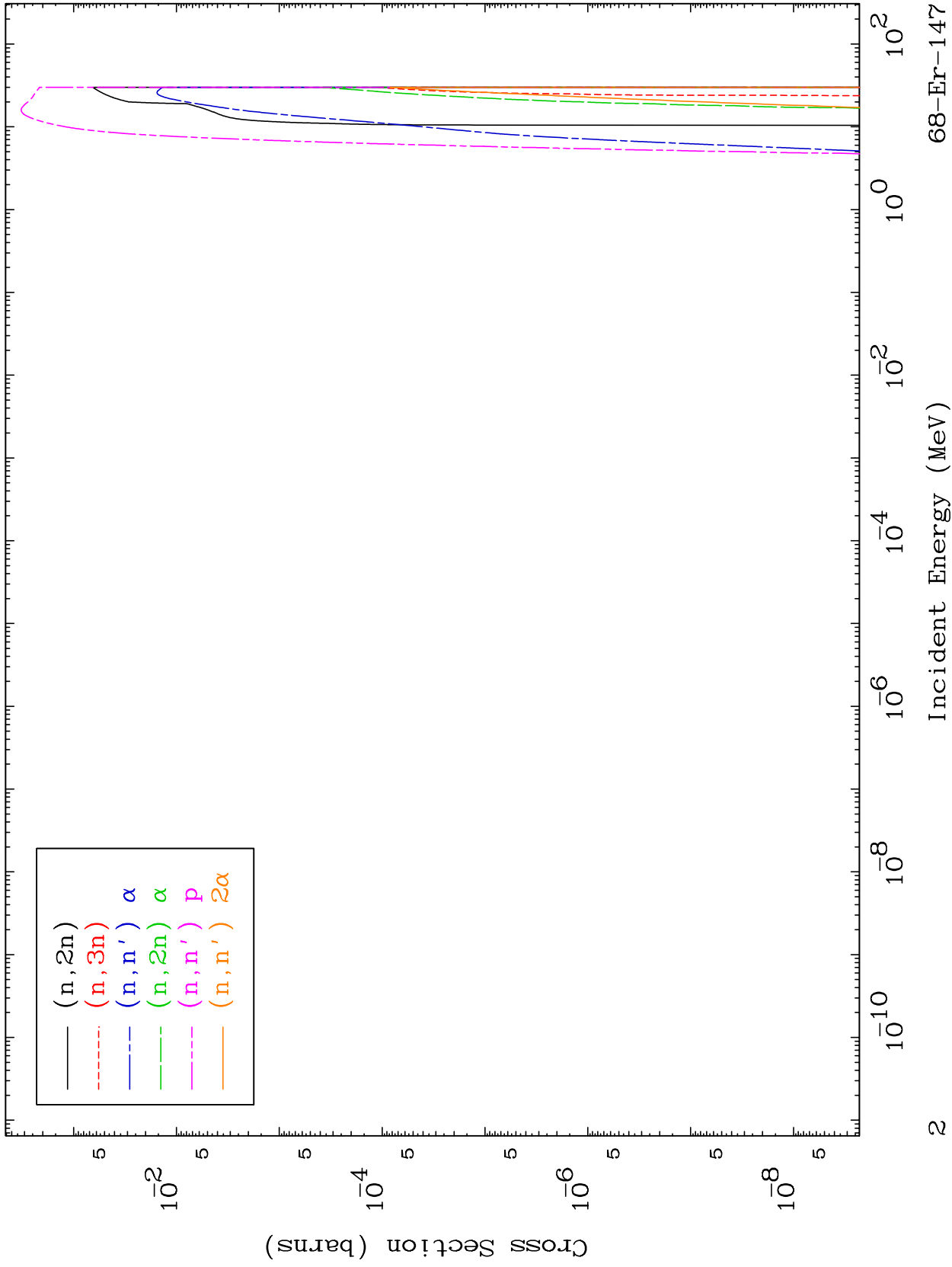
Incident Energy (MeV)

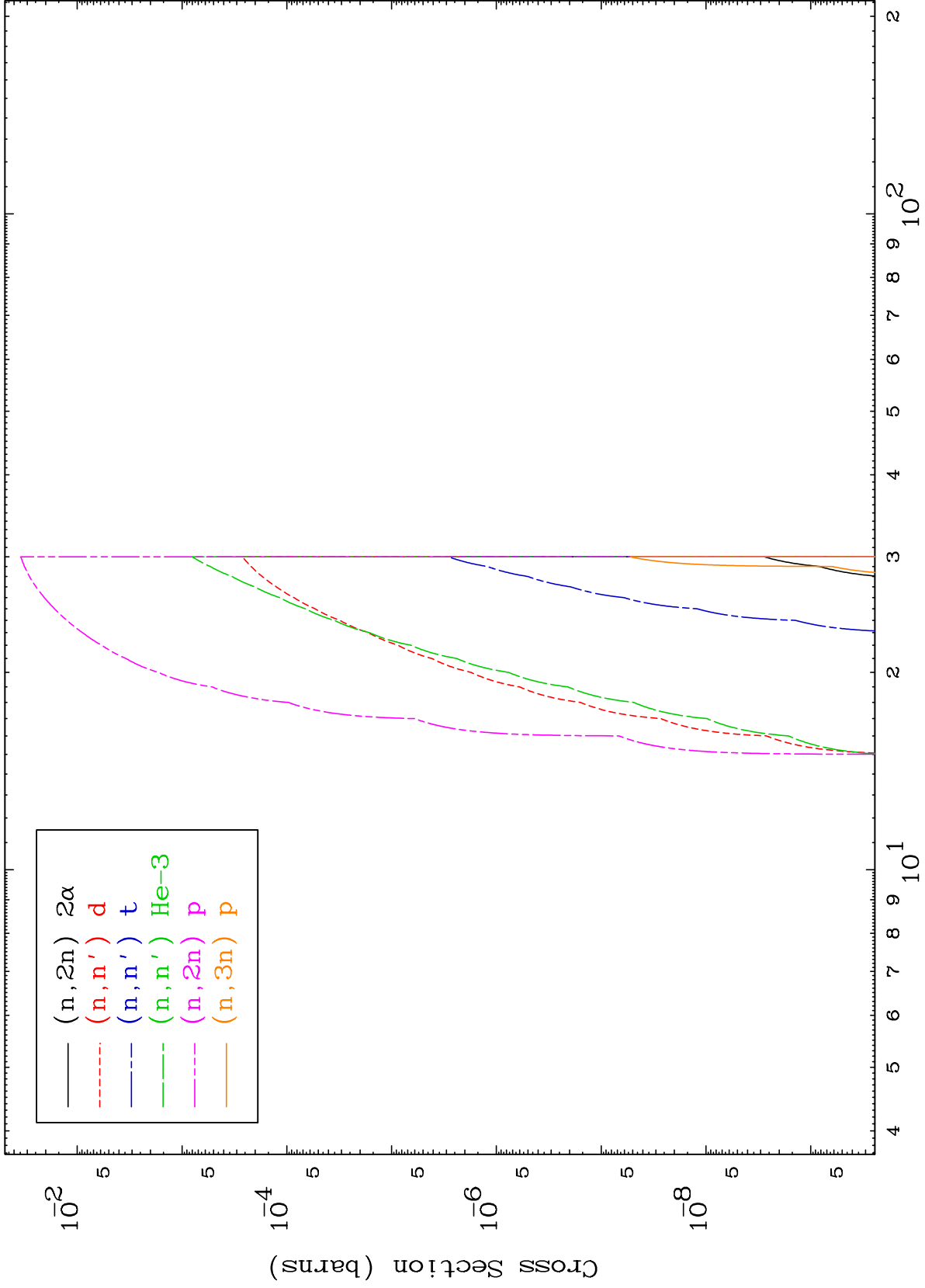
68-Er-147

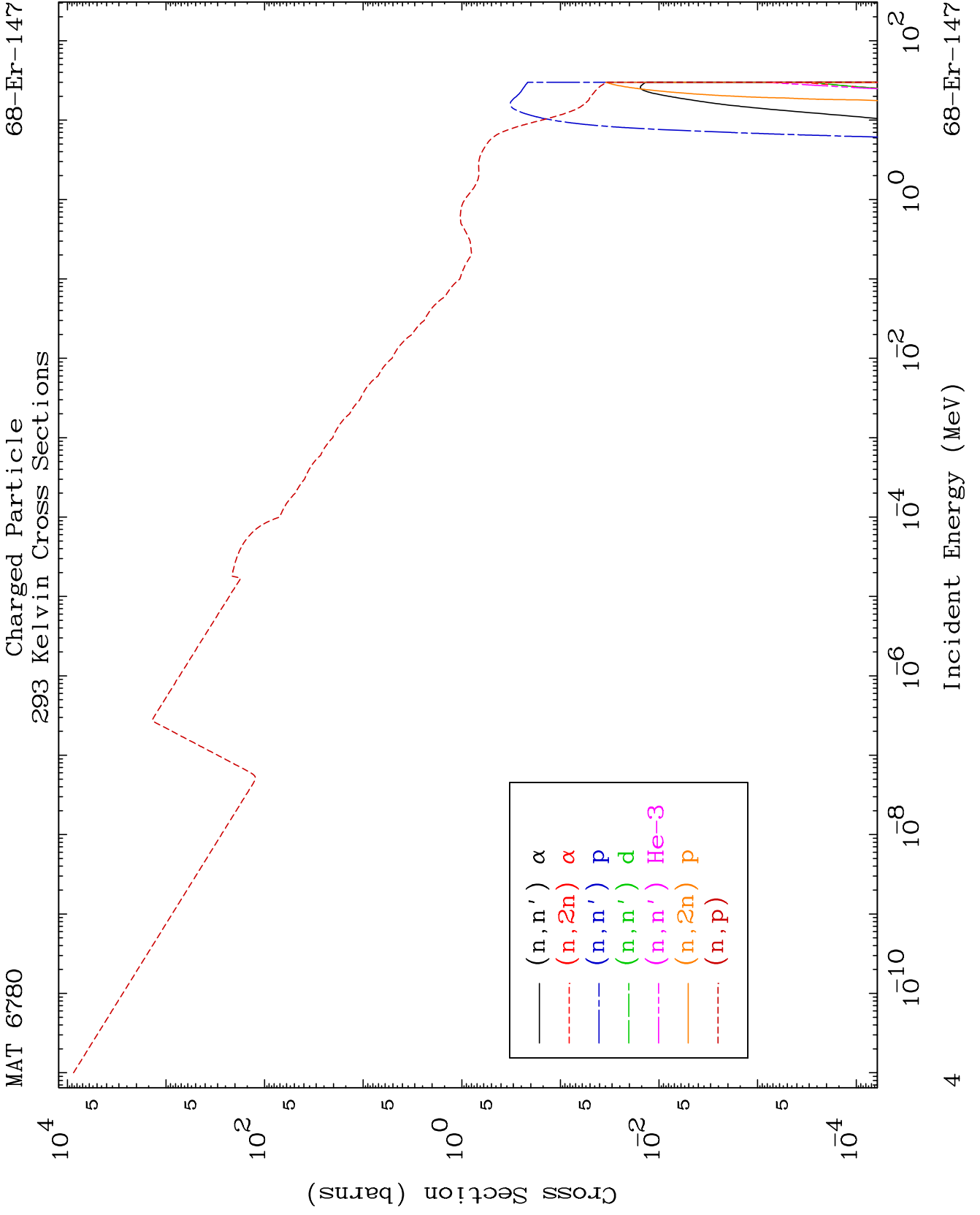
MAT 6780

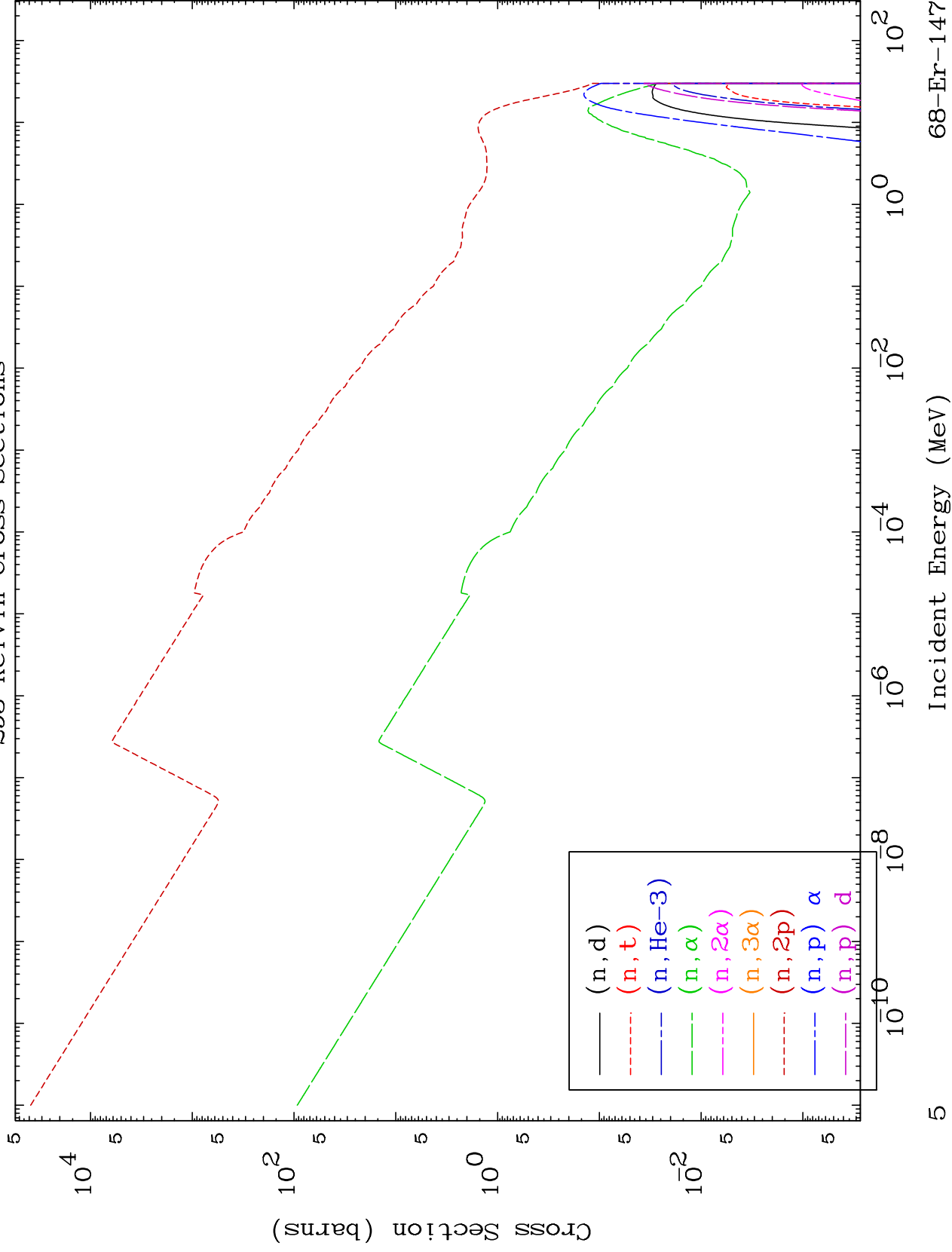
Neutron Production  
293 Kelvin Cross Sections

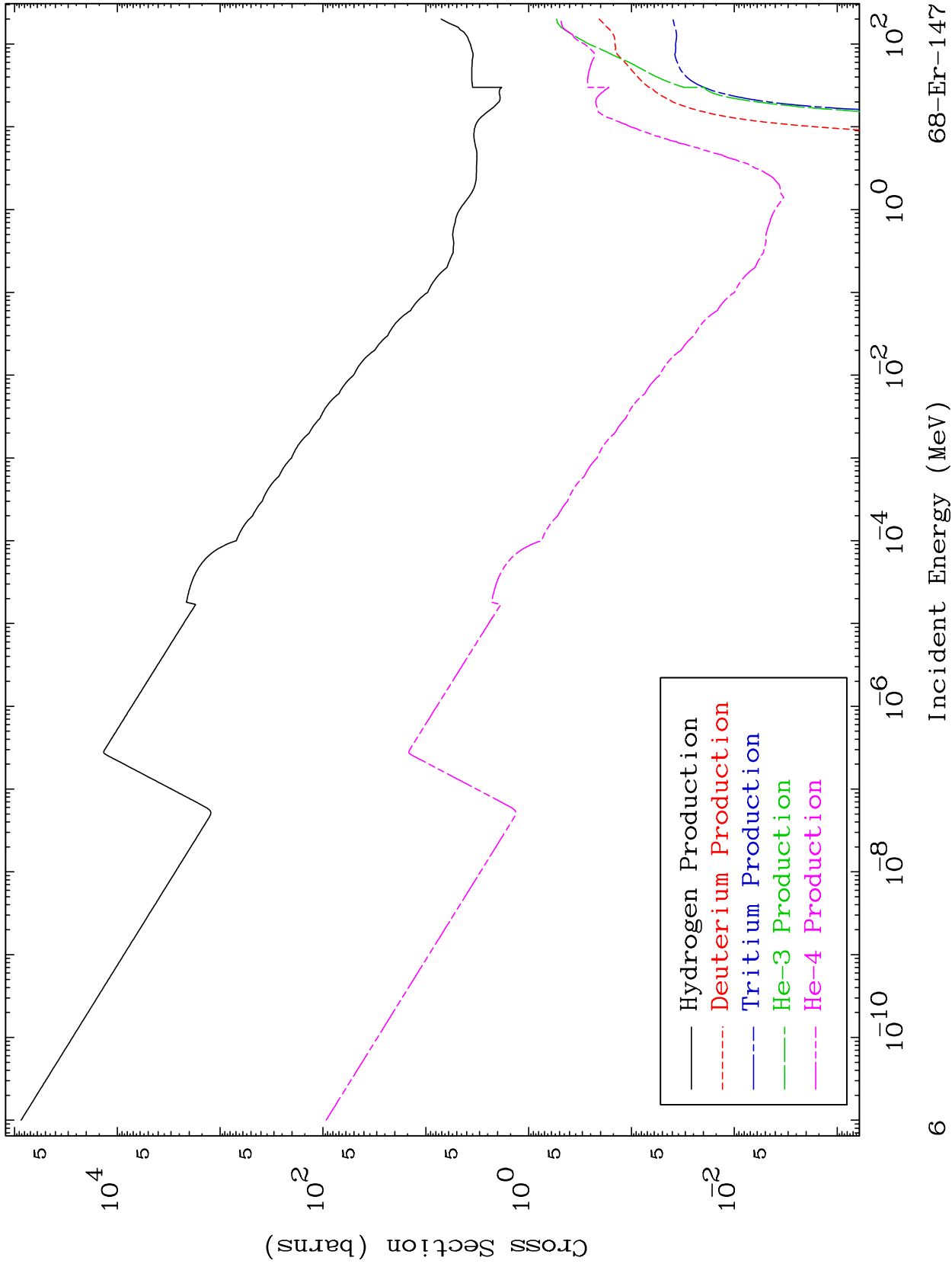
68-Er-147



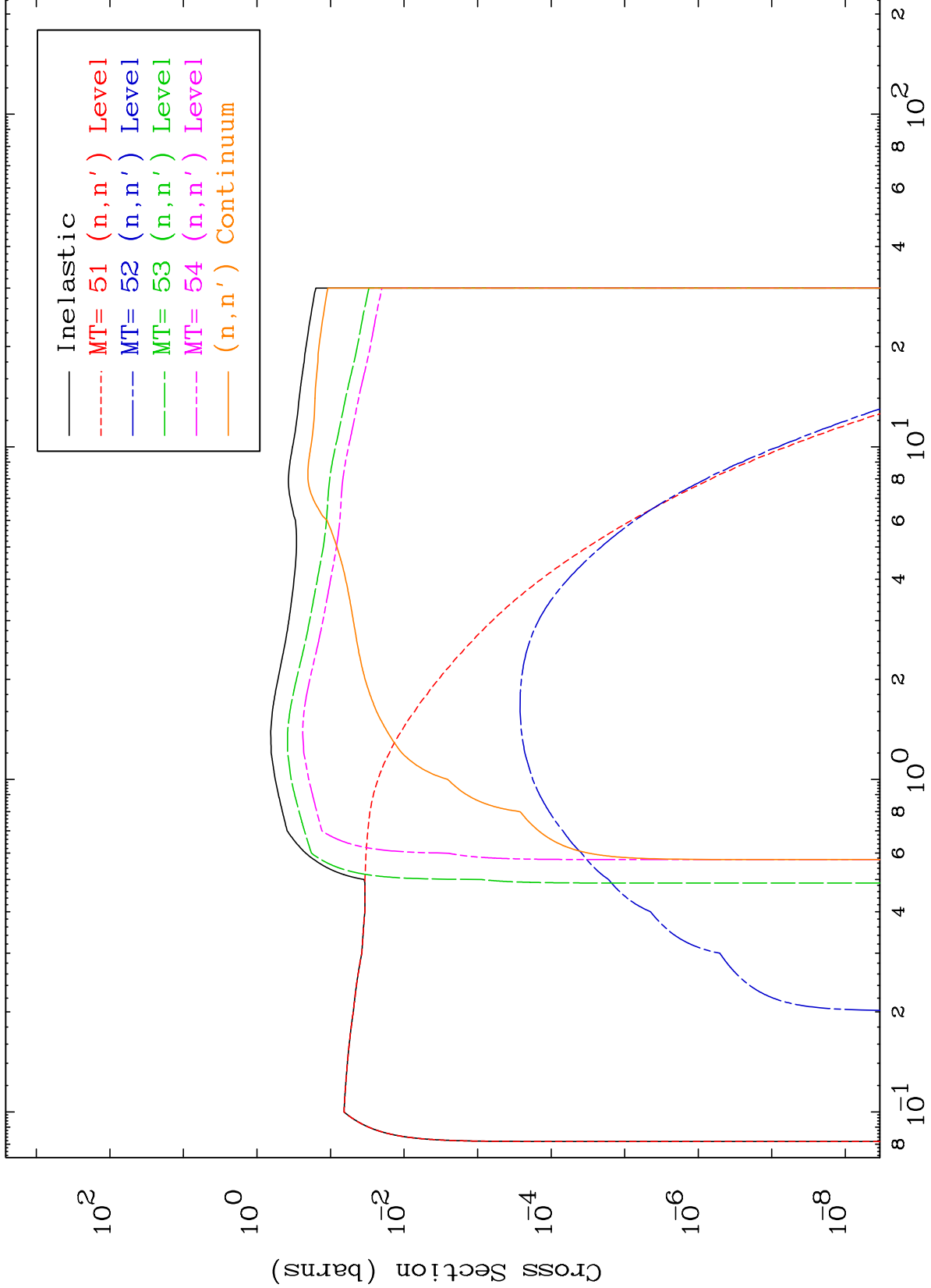




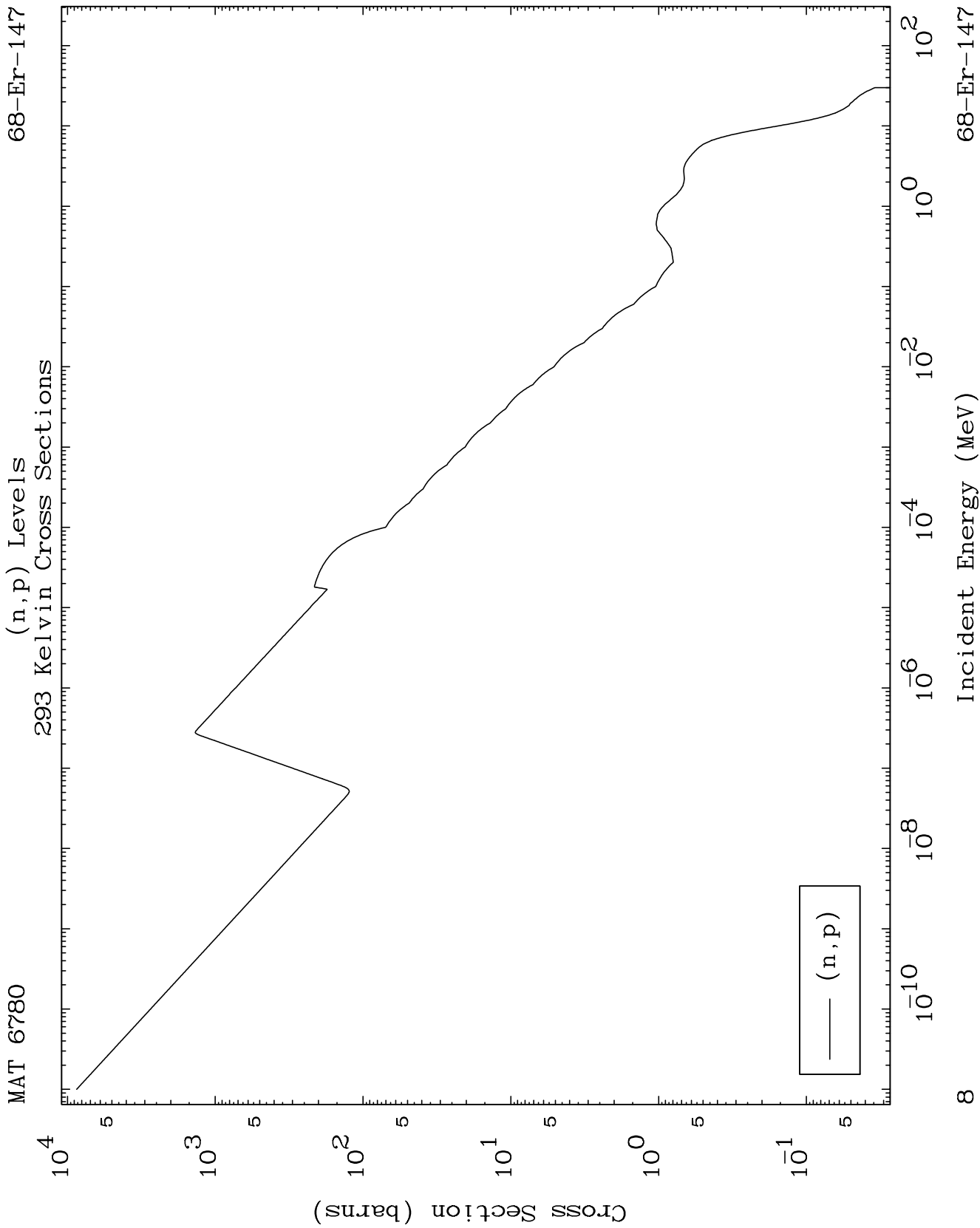




293 Kelvin Cross Sections



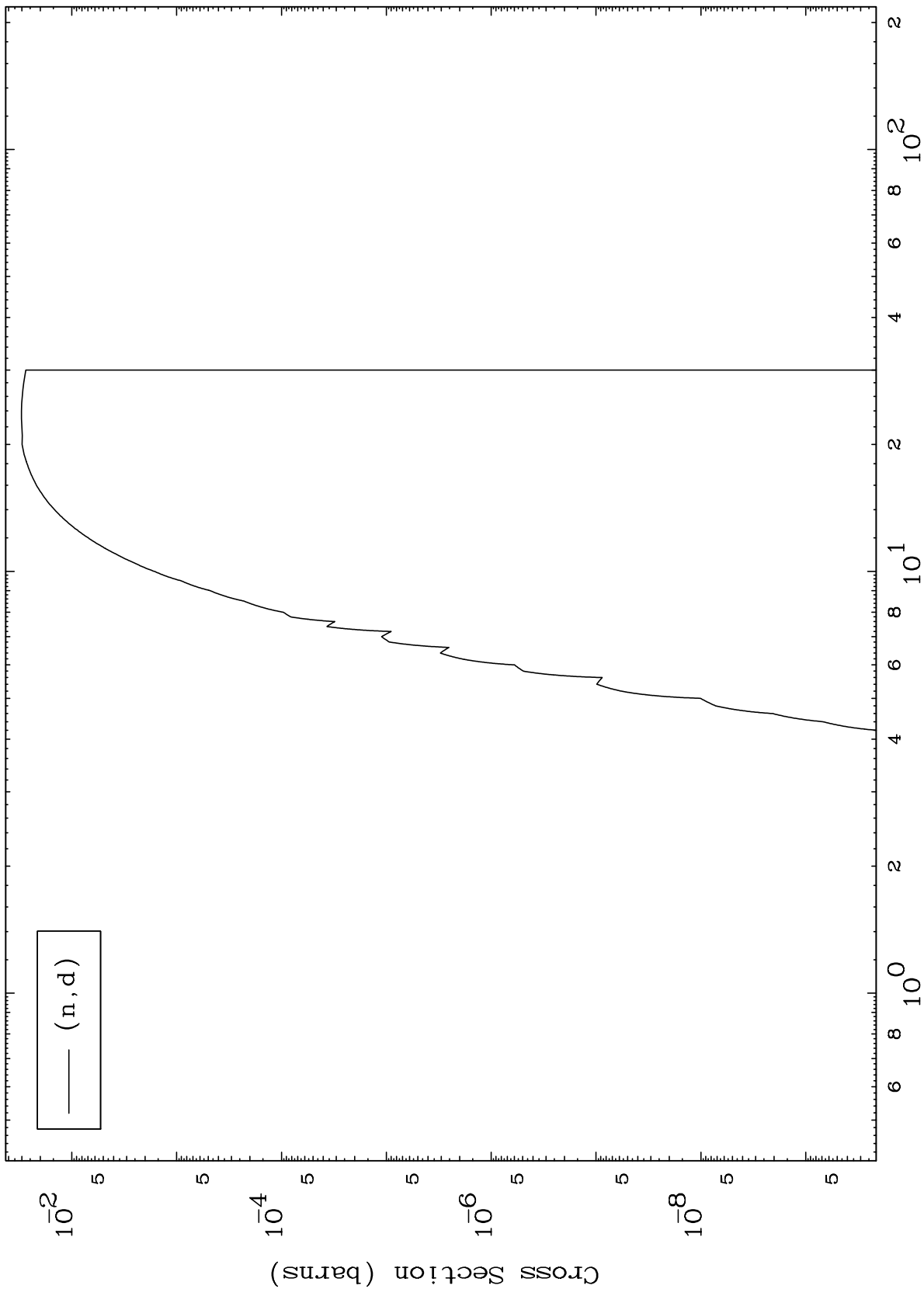




MAT 6780

(n,d) Levels  
293 Kelvin Cross Sections

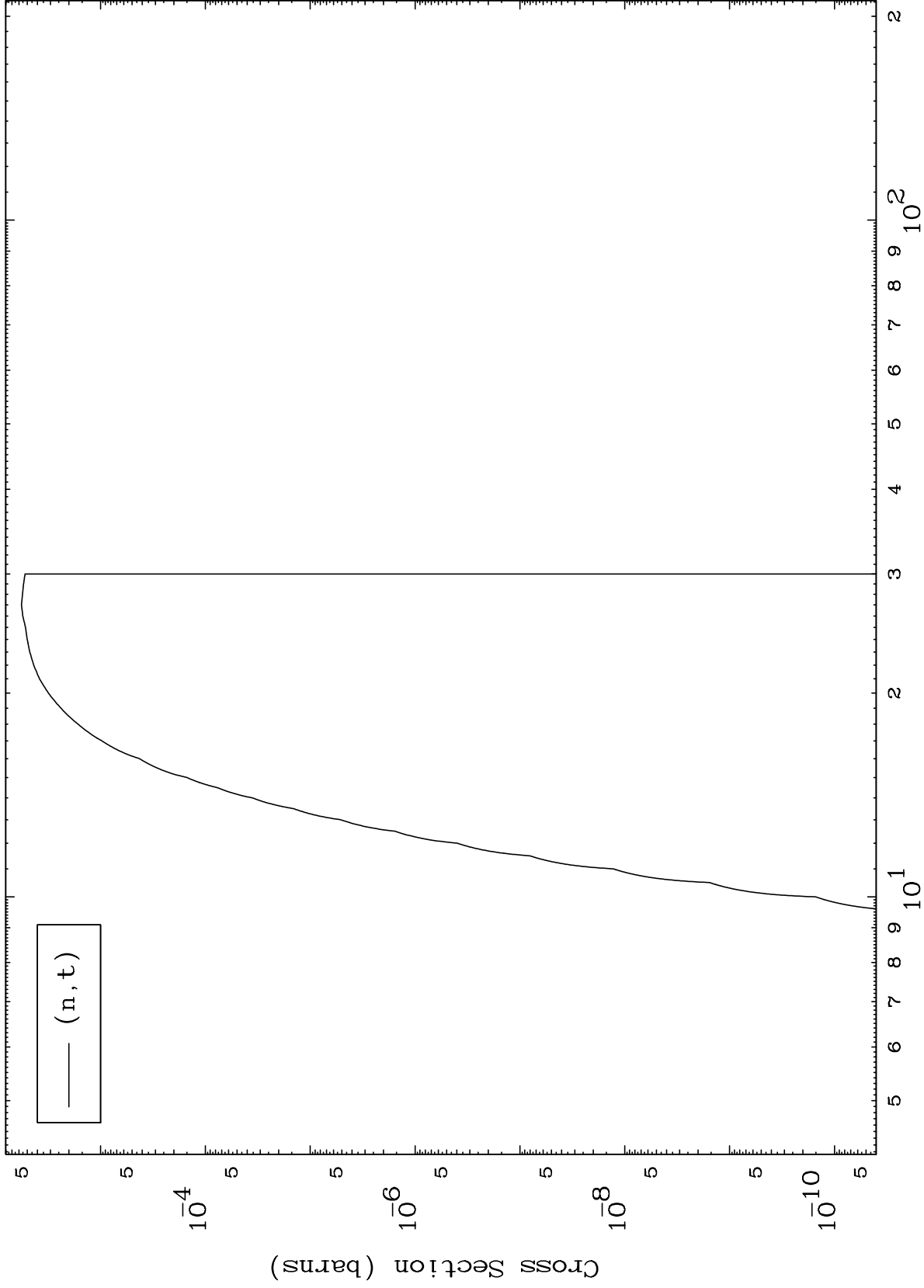
68-Er-147



MAT 6780

(n,t) Levels  
293 Kelvin Cross Sections

68-Er-147



10

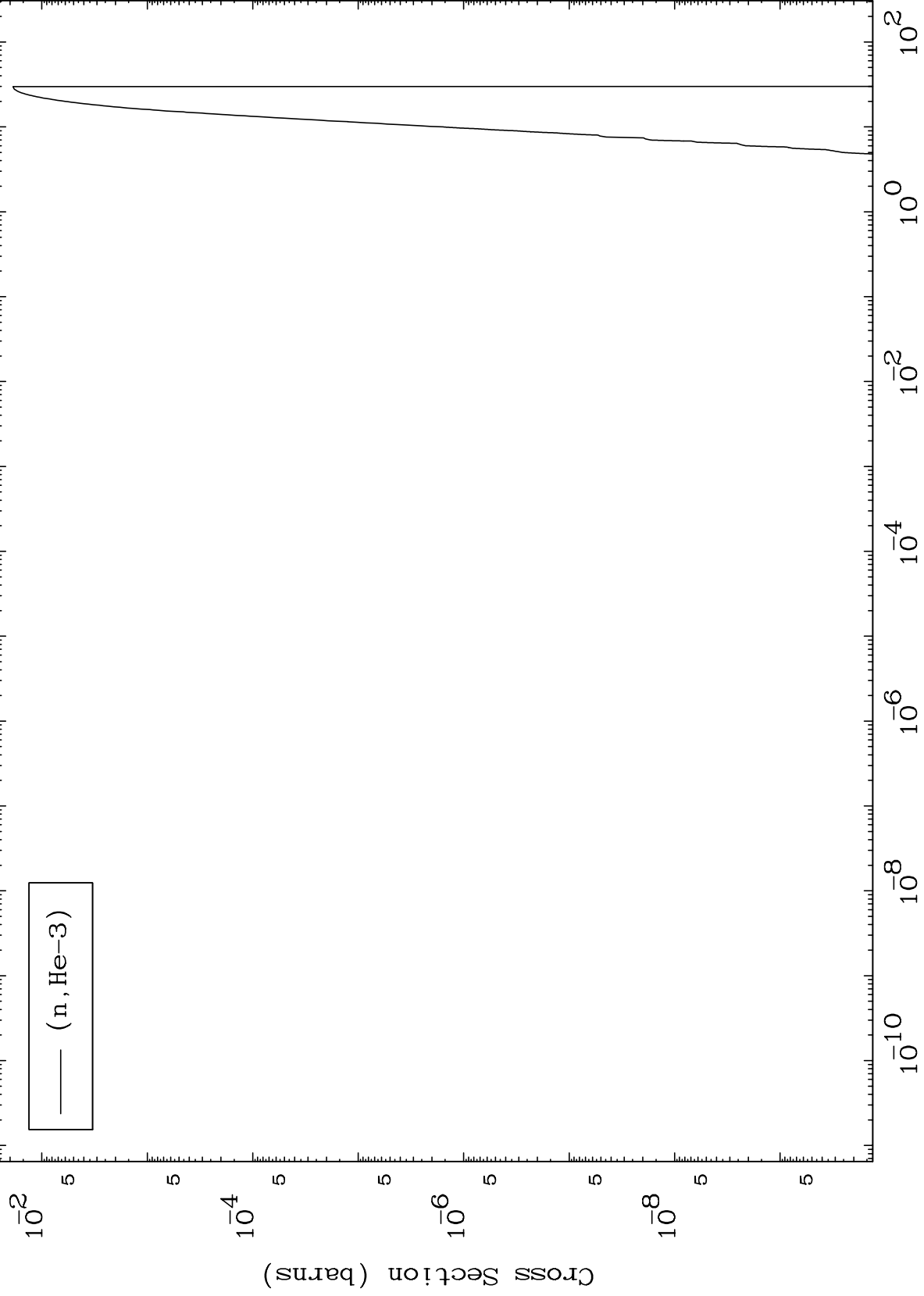
Incident Energy (MeV)

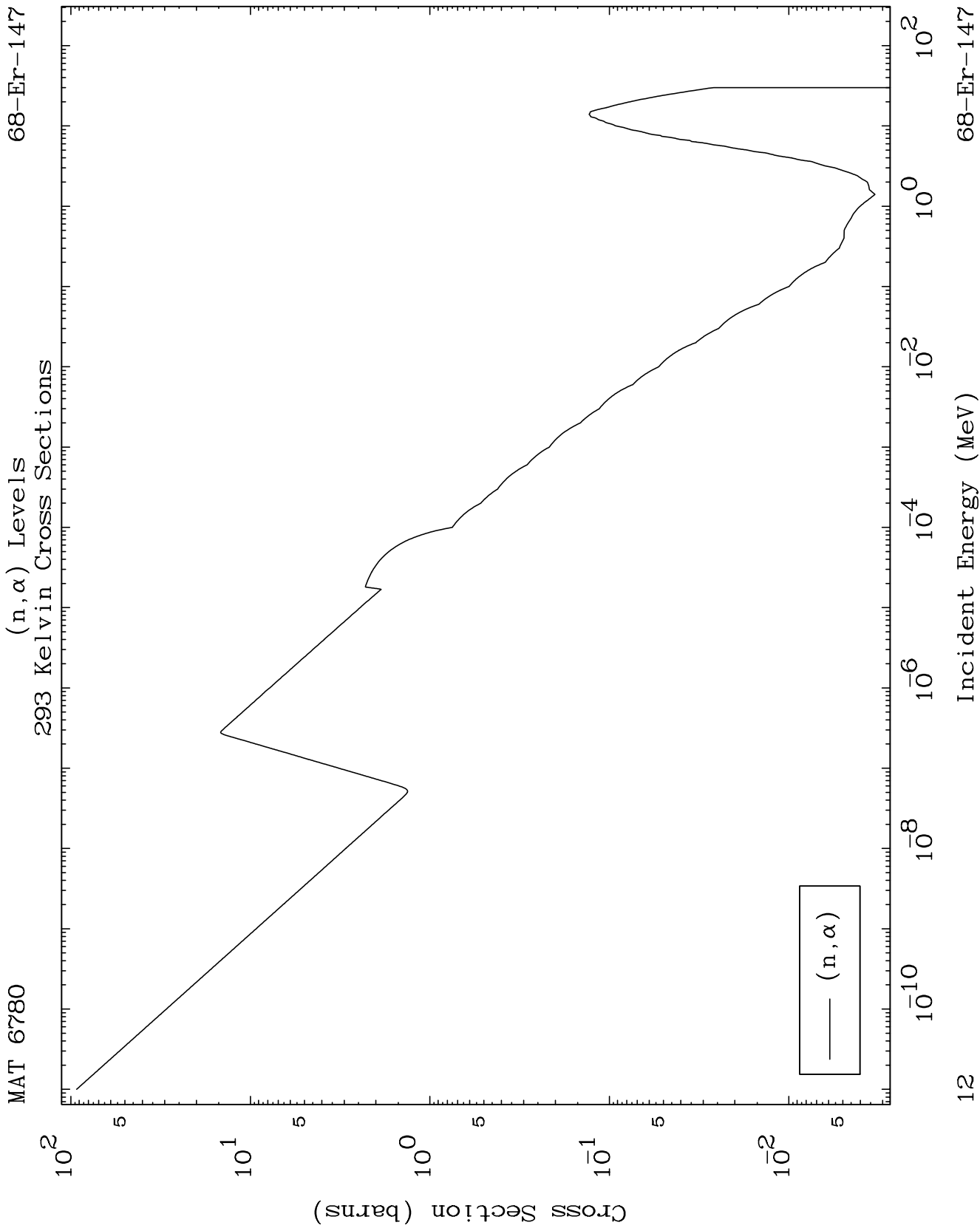
68-Er-147

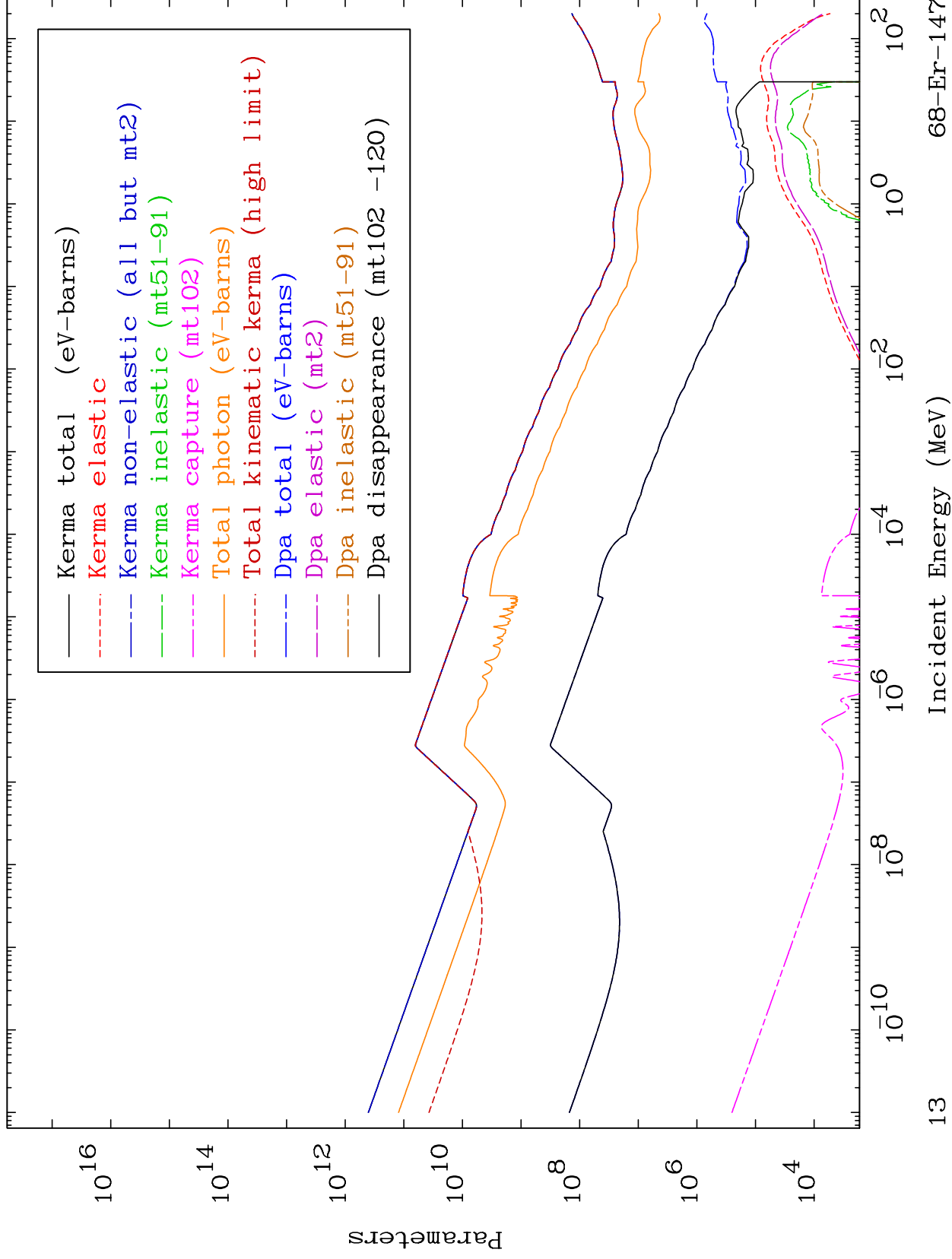
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(n,He3) Levels  
293 Kelvin Cross Sections

68-Er-147



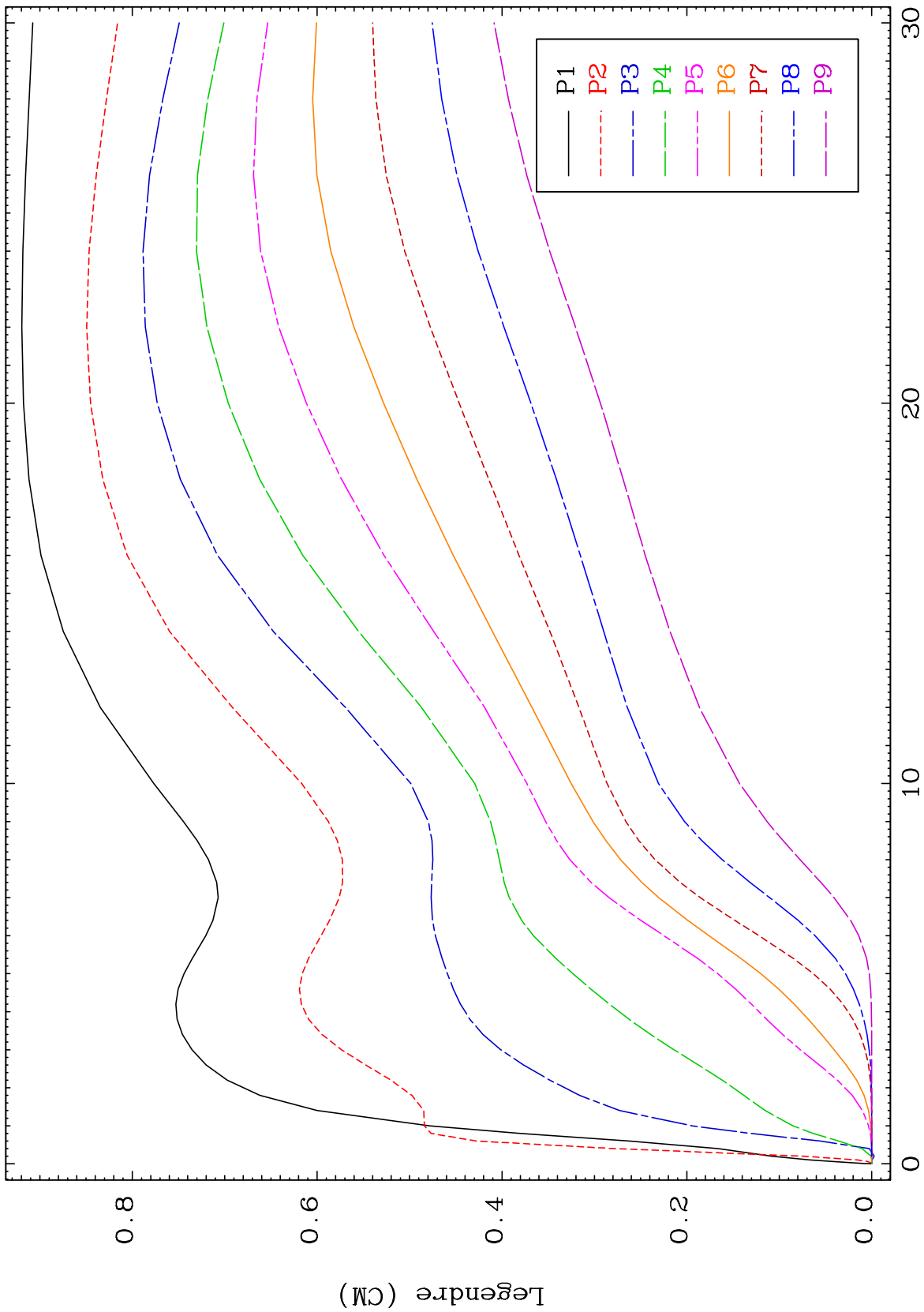




MAT 6780

68-Er-147

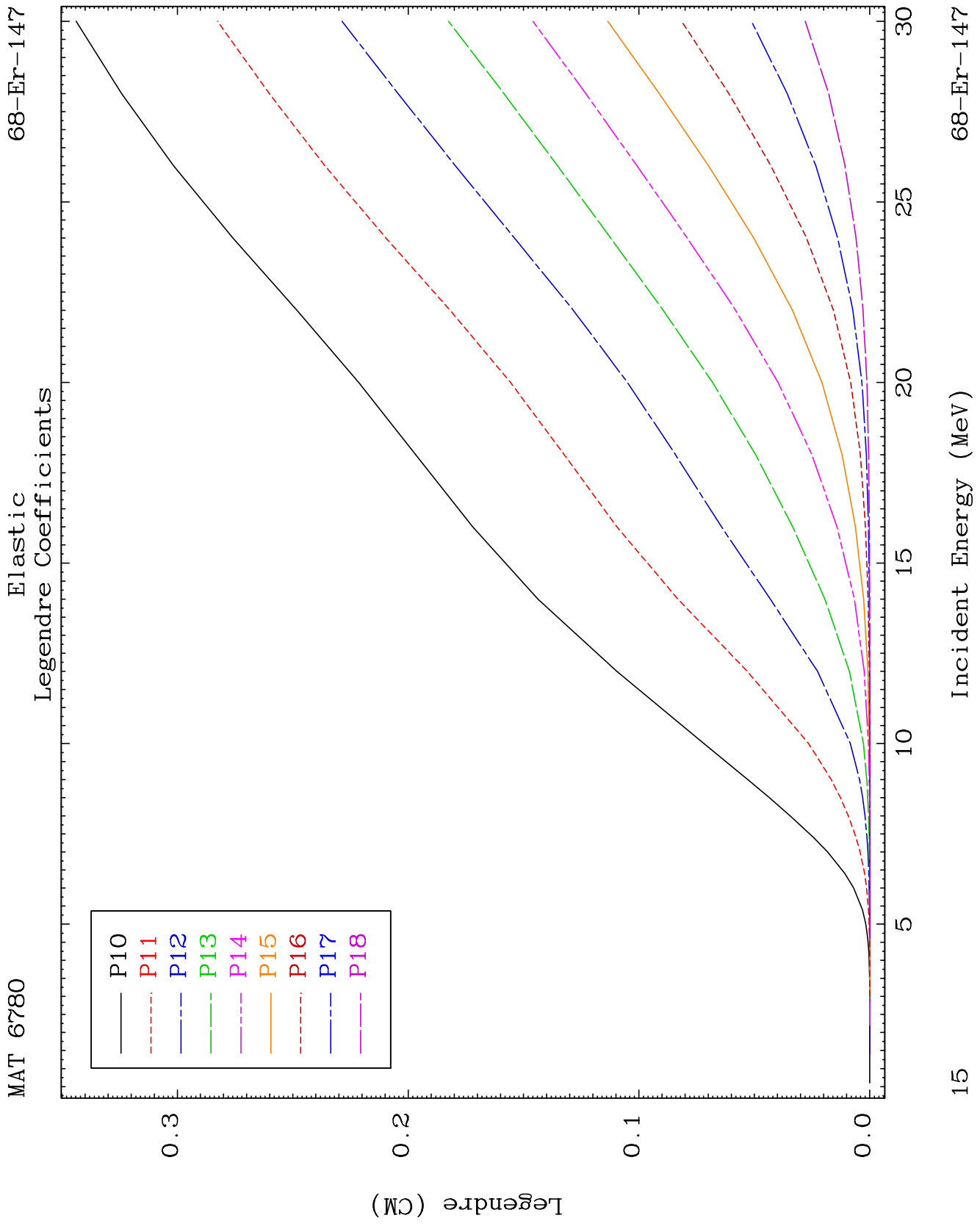
Elastic Legendre Coefficients



14

Incident Energy (MeV)

68-Er-147

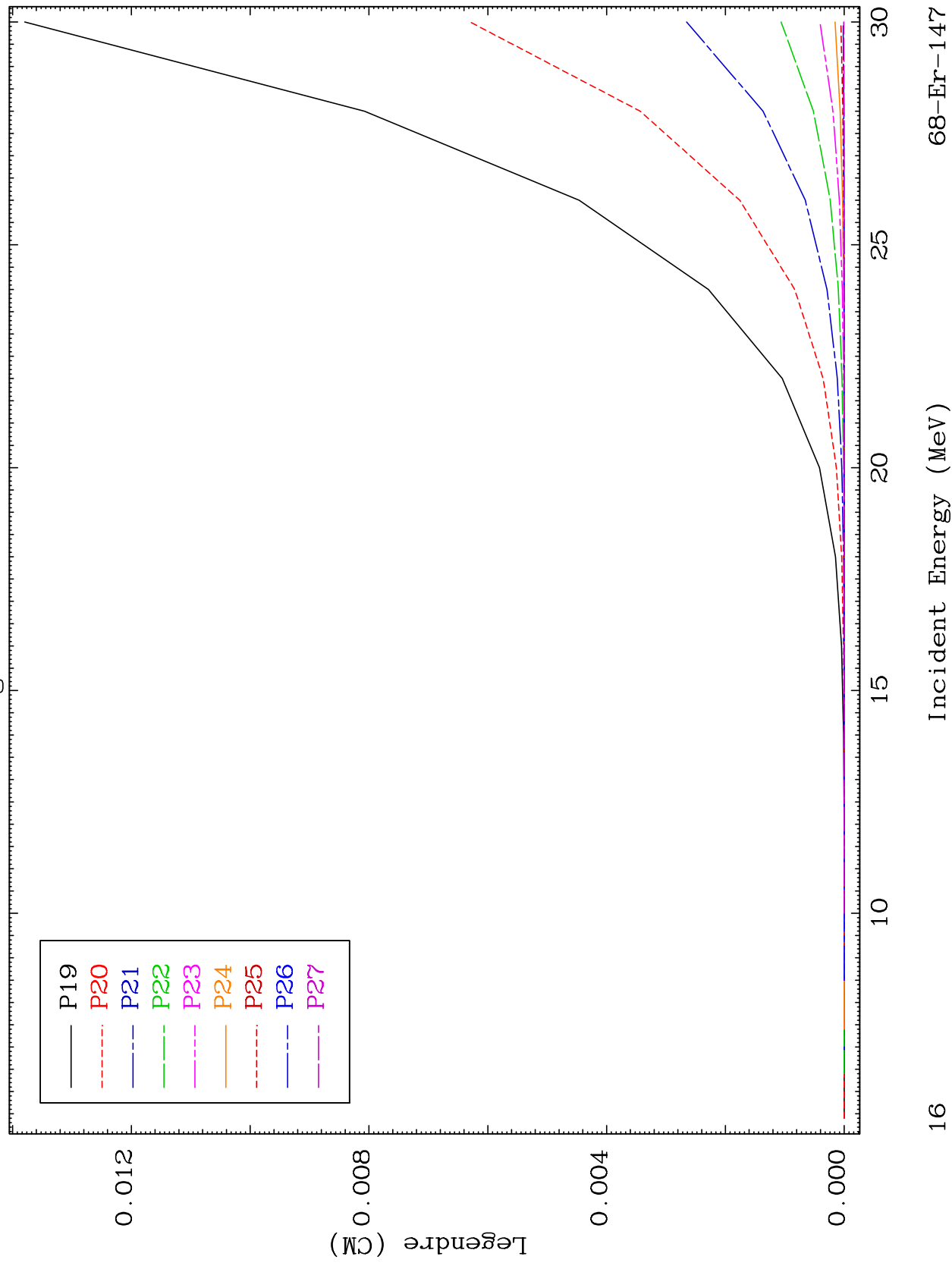




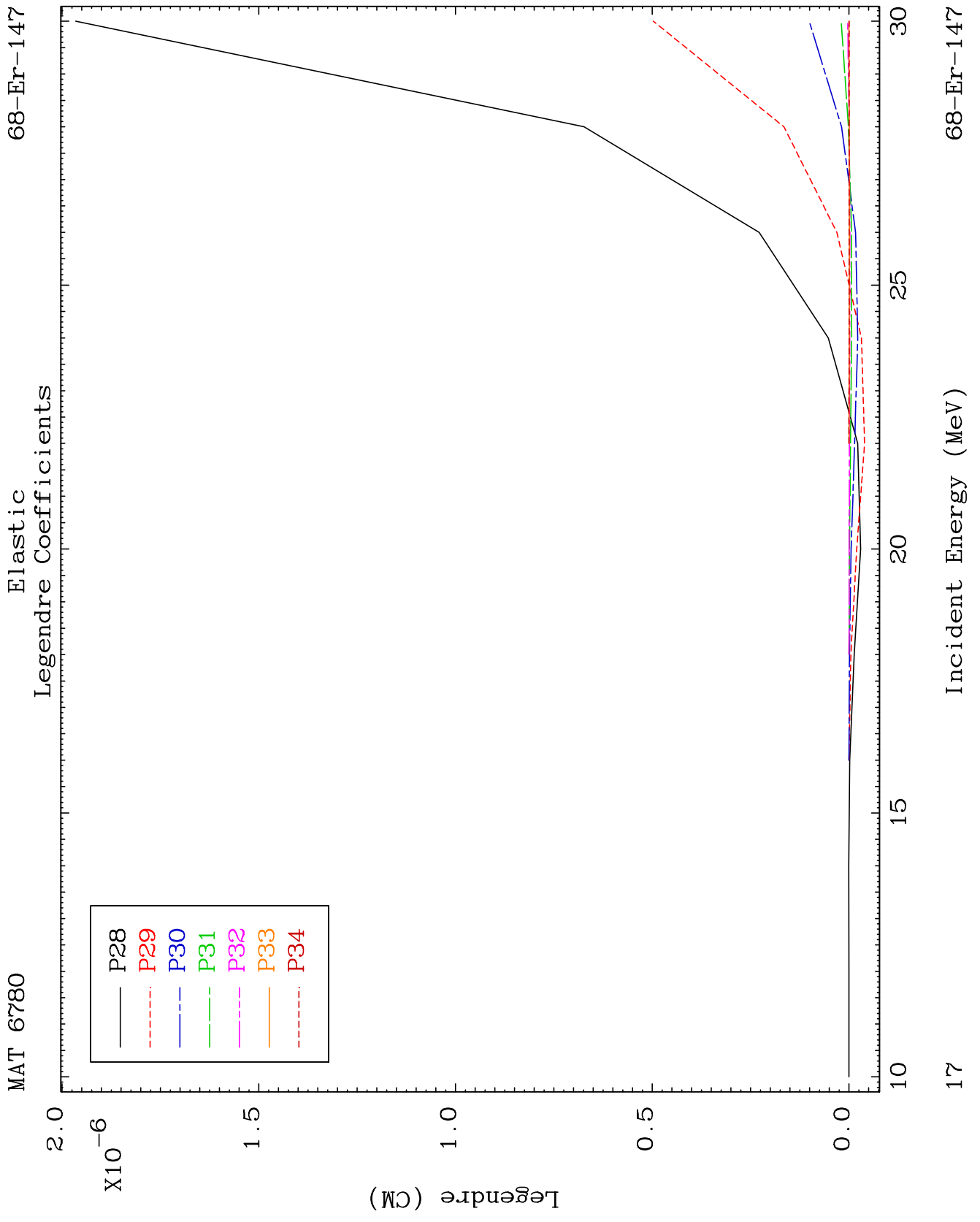
MAT 6780

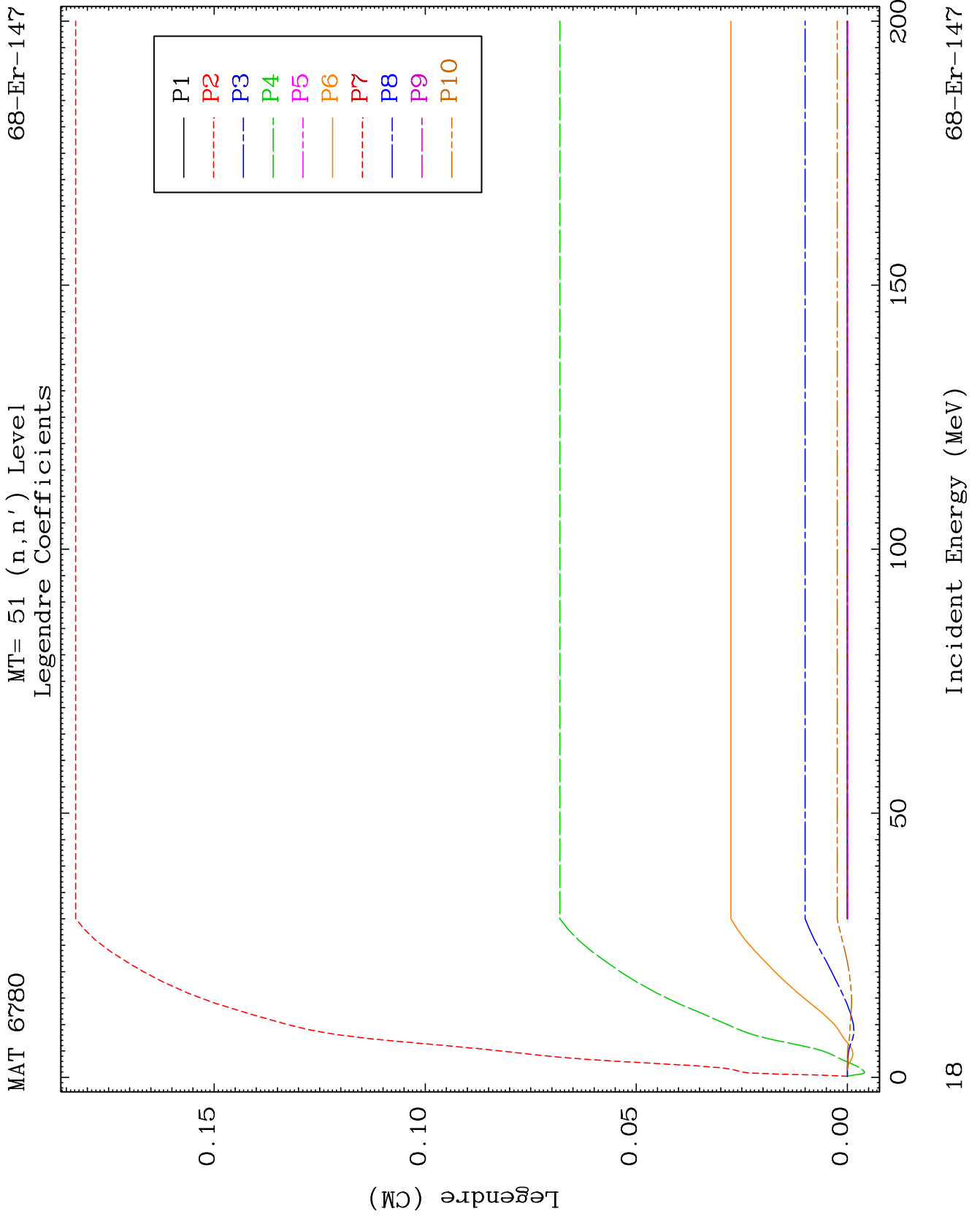
### Elastic Legendre Coefficients

68-Er-147



- P19
- P20
- P21
- P22
- P23
- P24
- P25
- P26
- P27

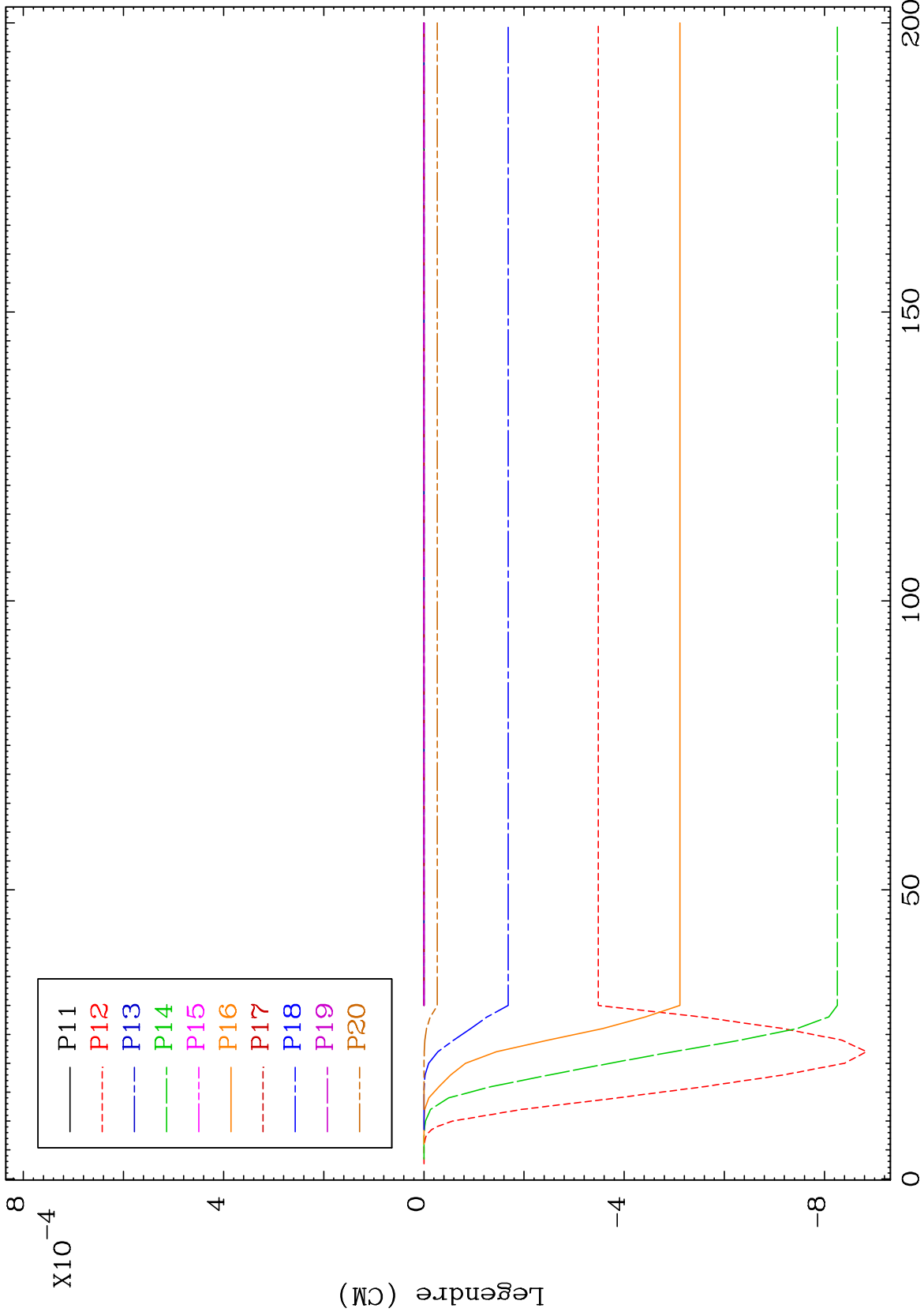




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MT= 51 (n,n') Level  
Legendre Coefficients

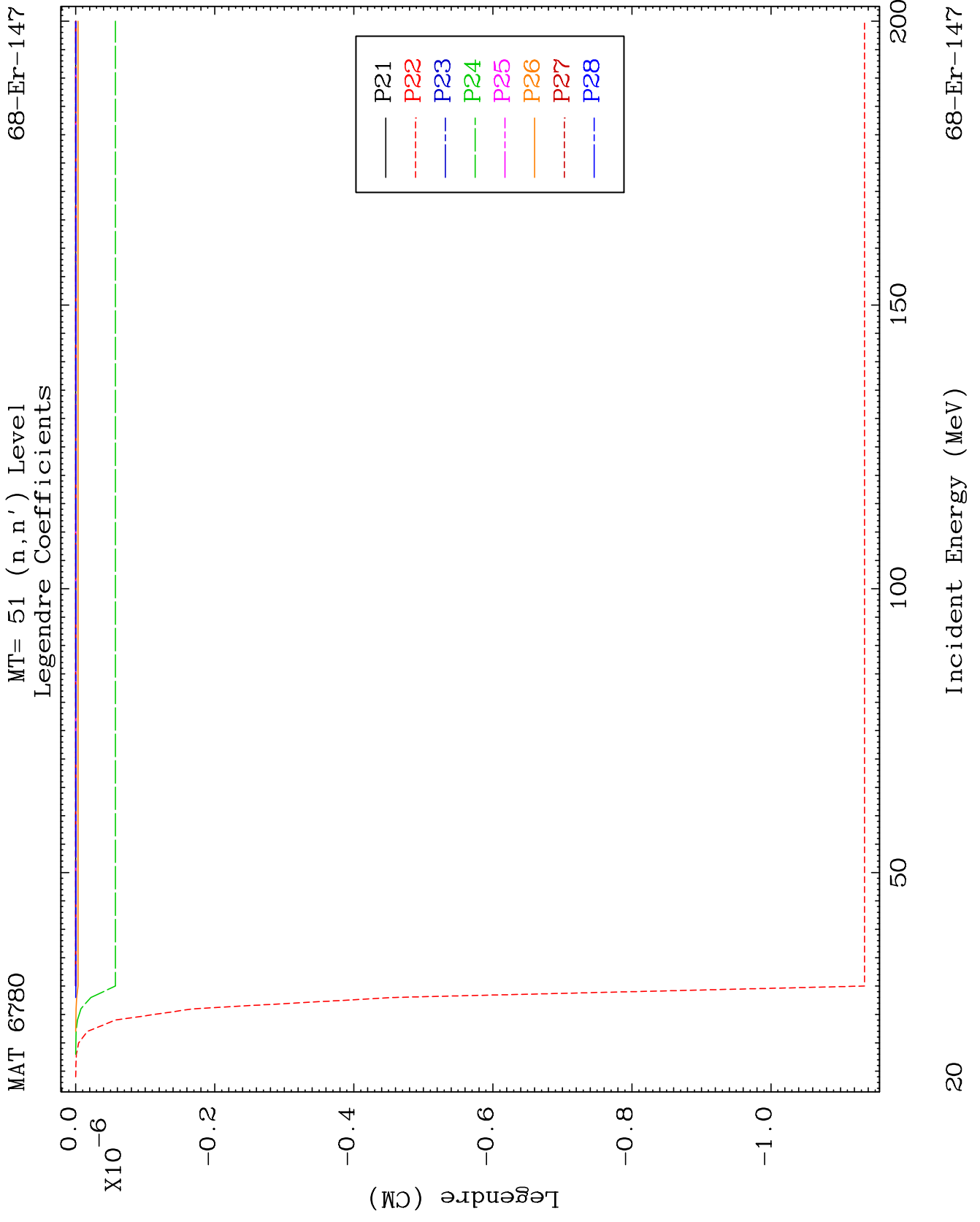
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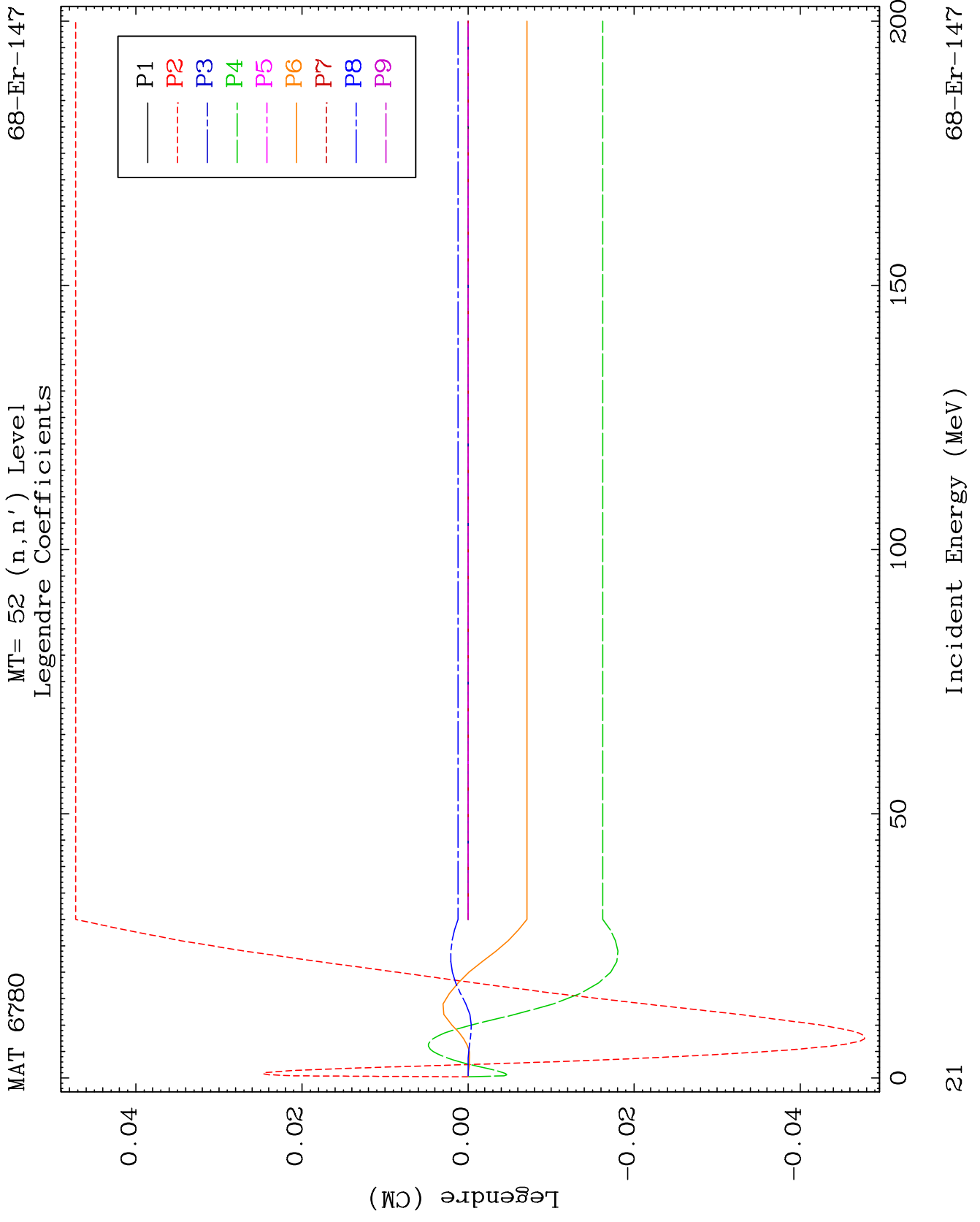


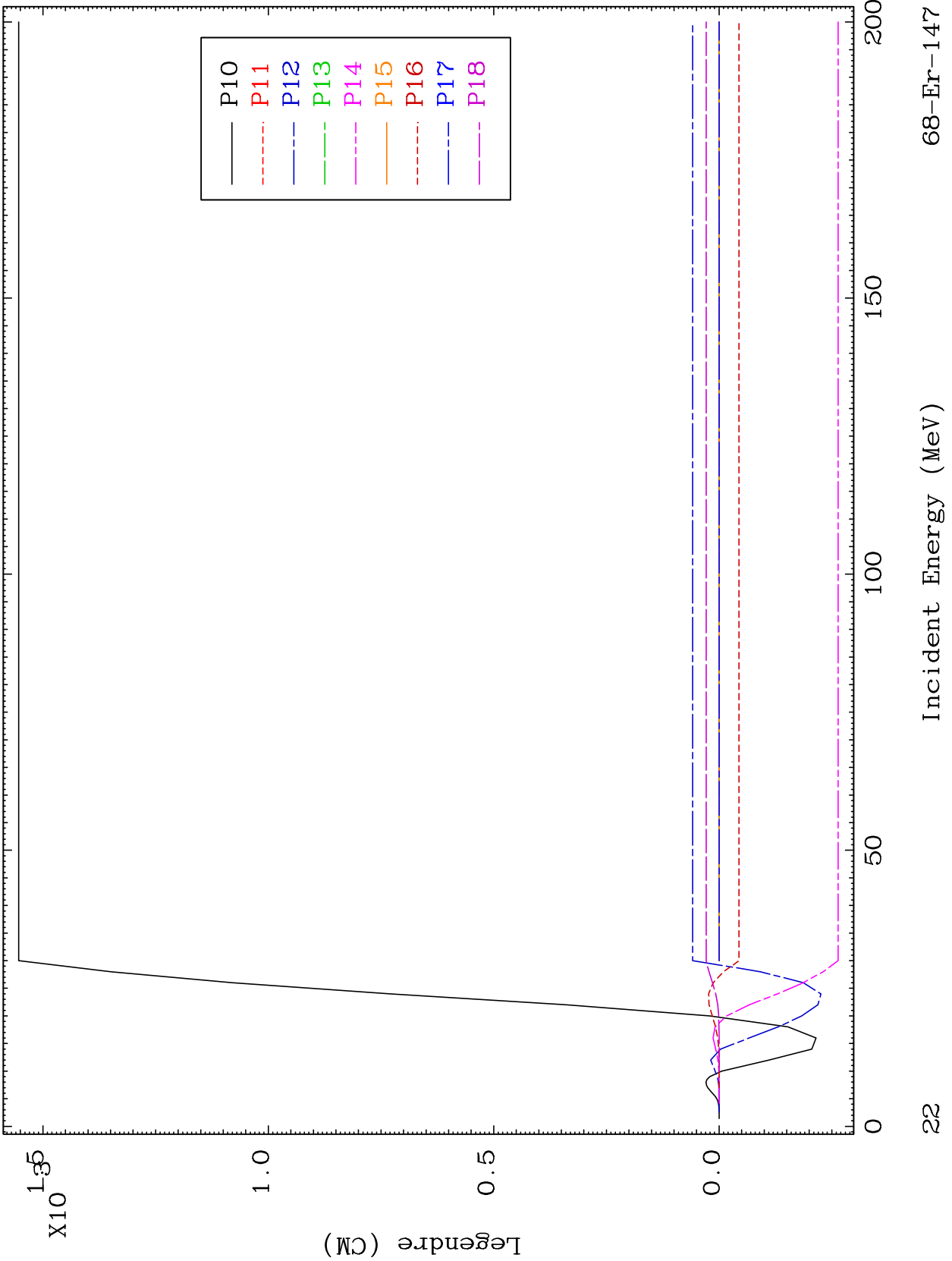
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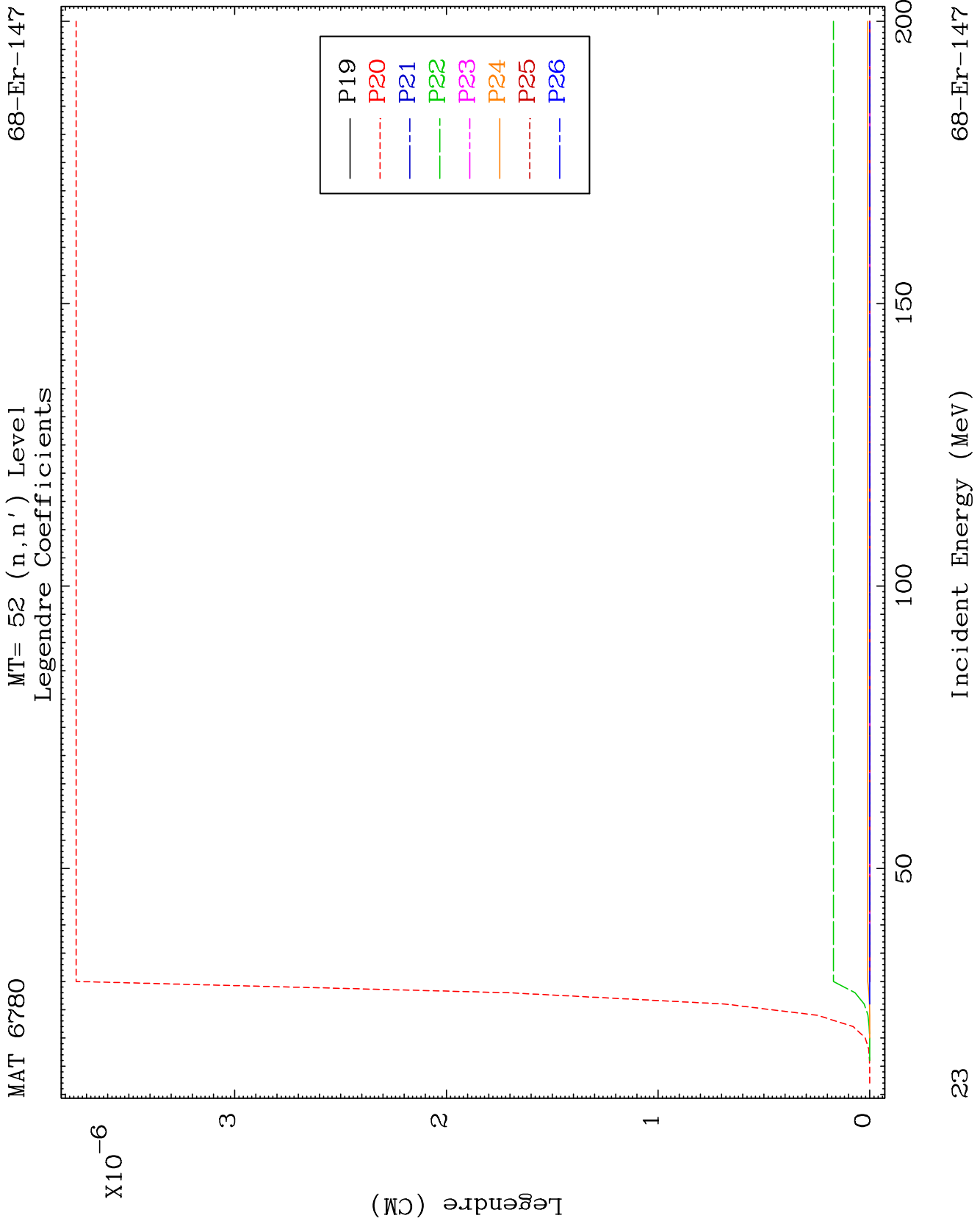
Incident Energy (MeV)

68-Er-147

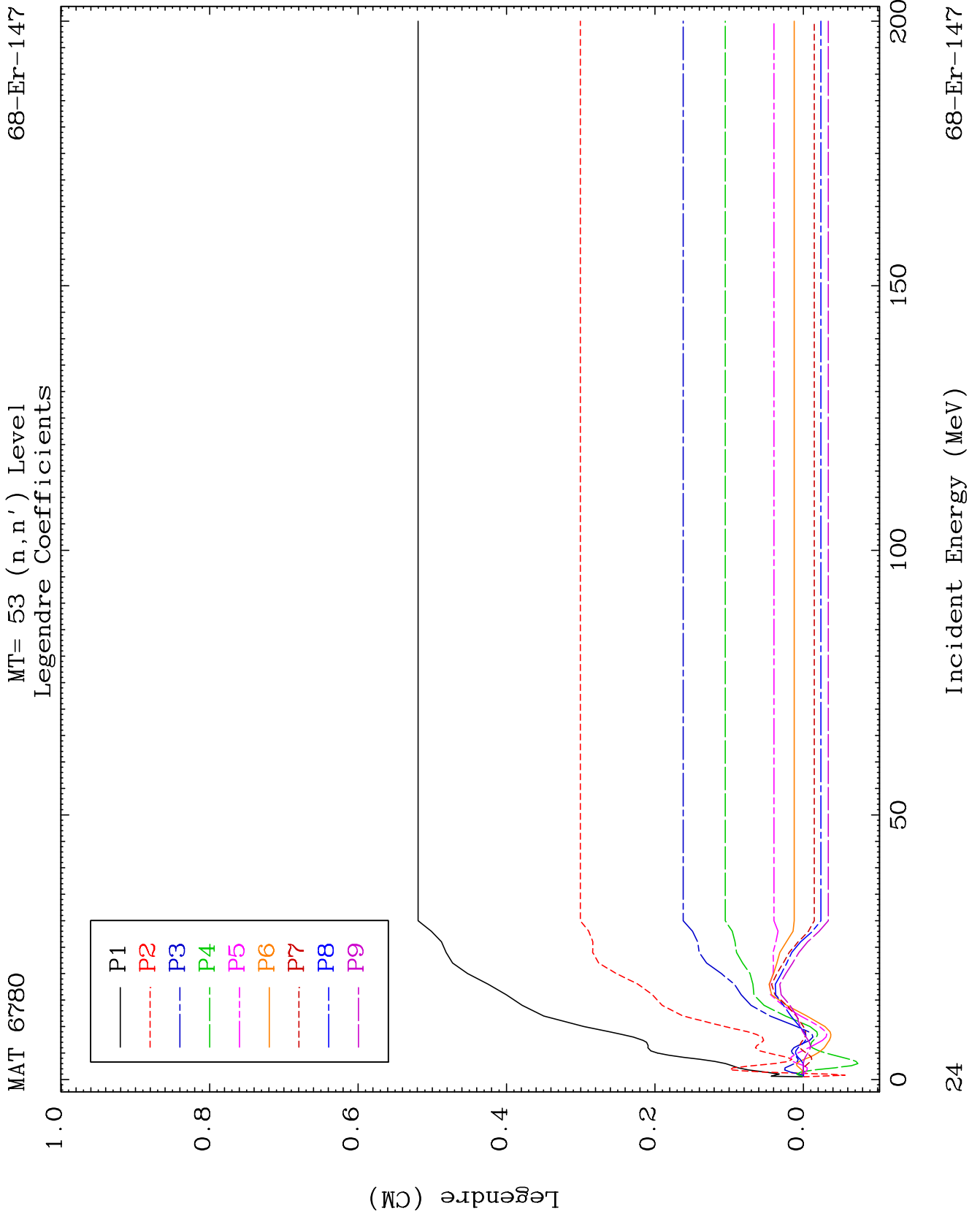


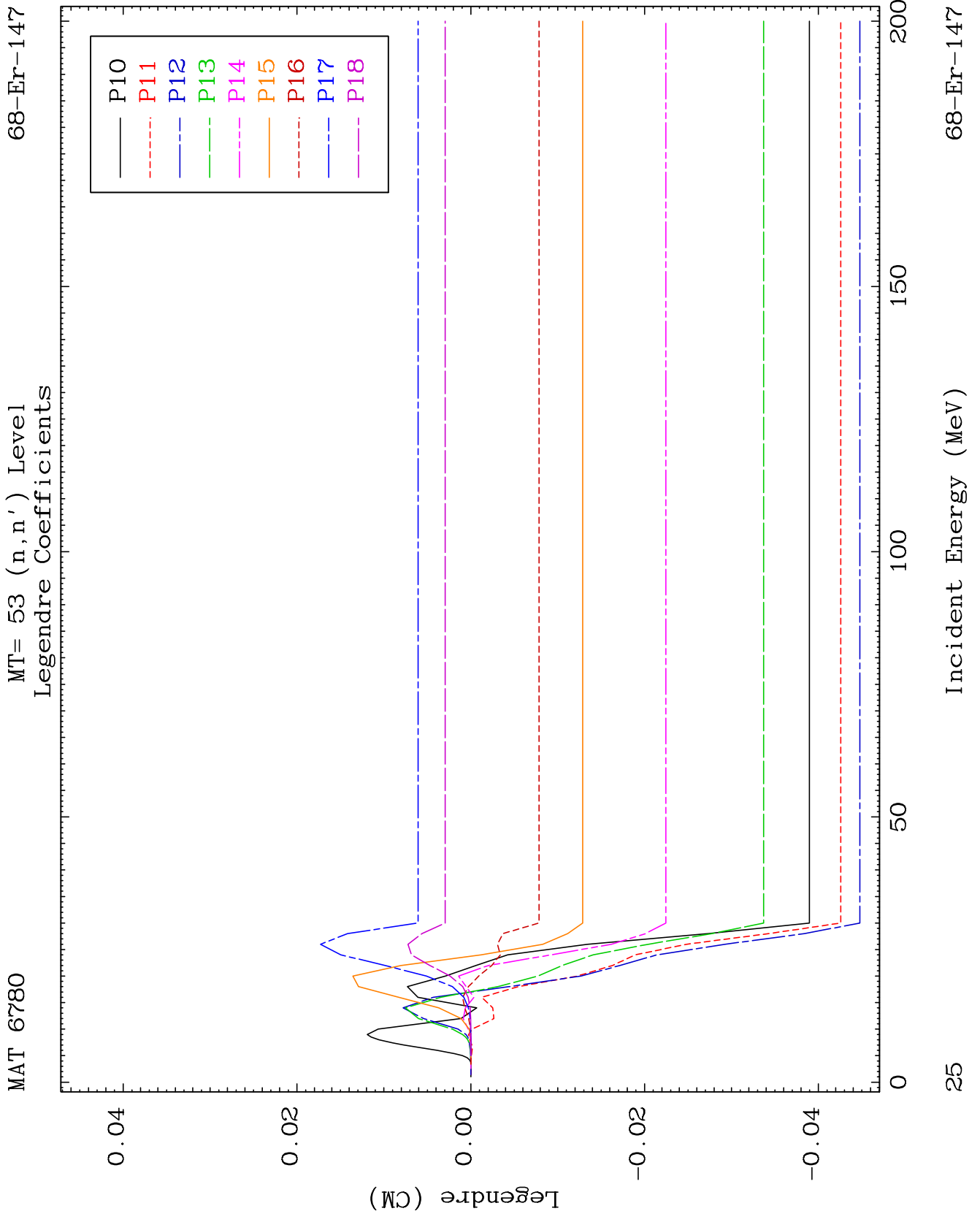


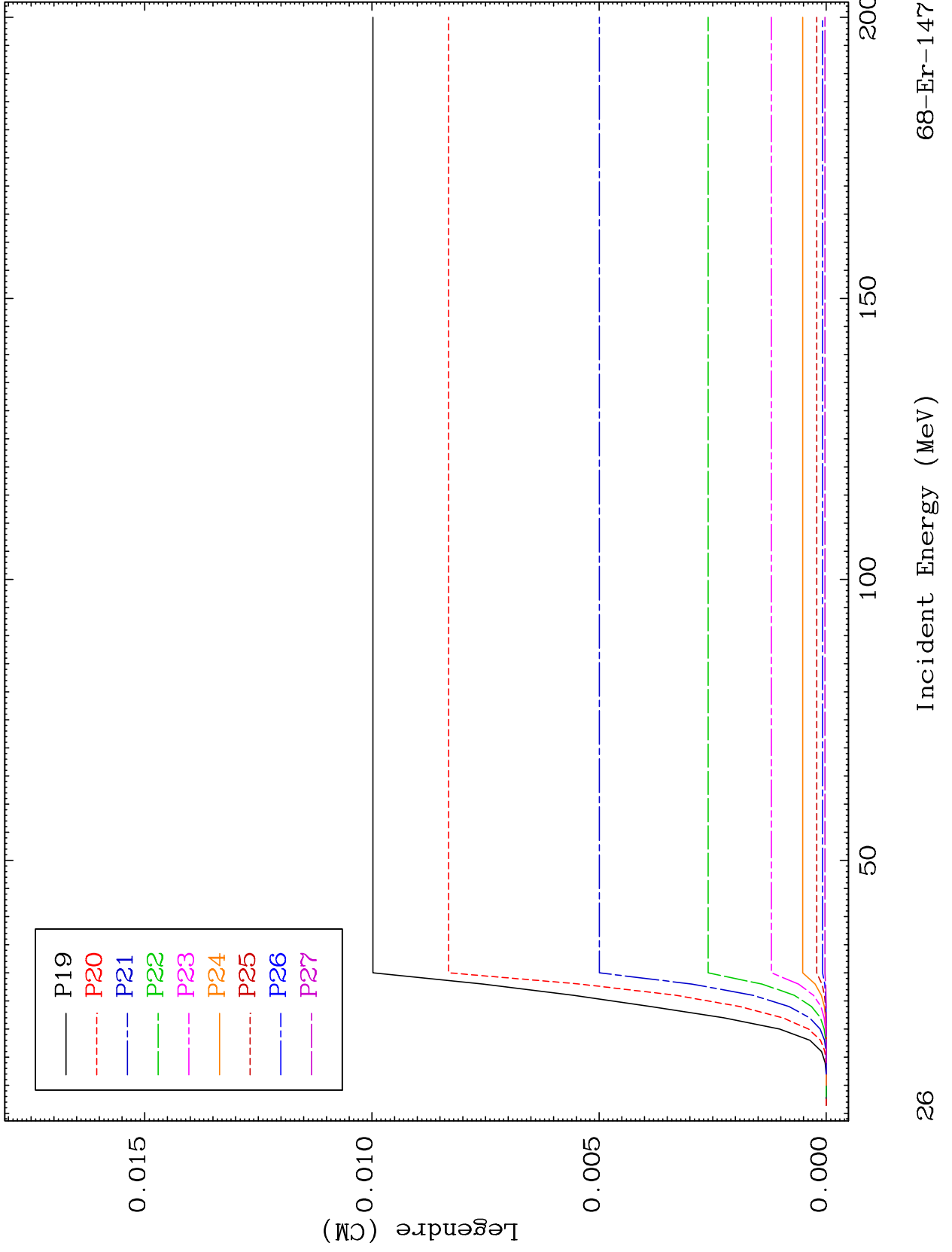


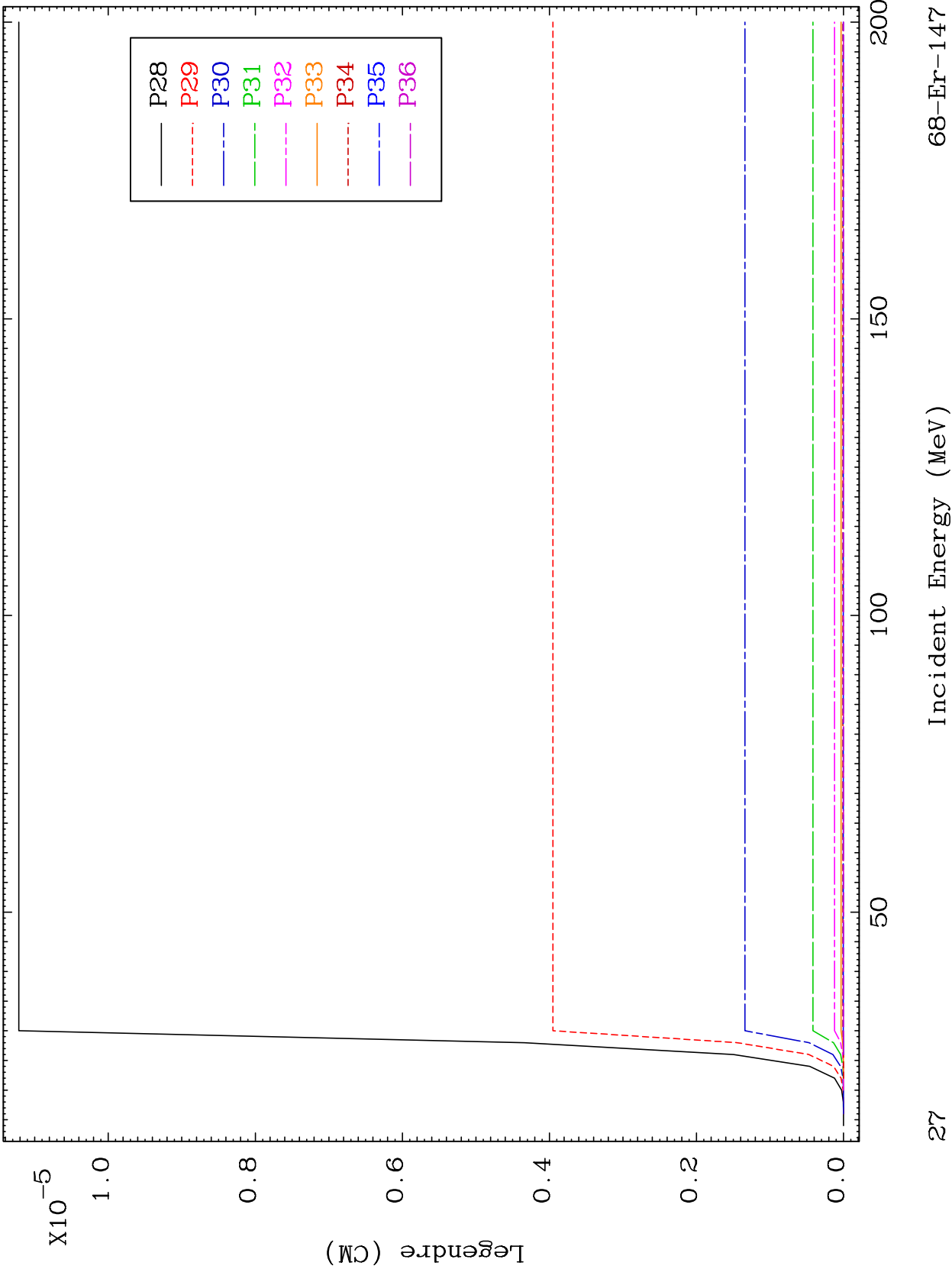








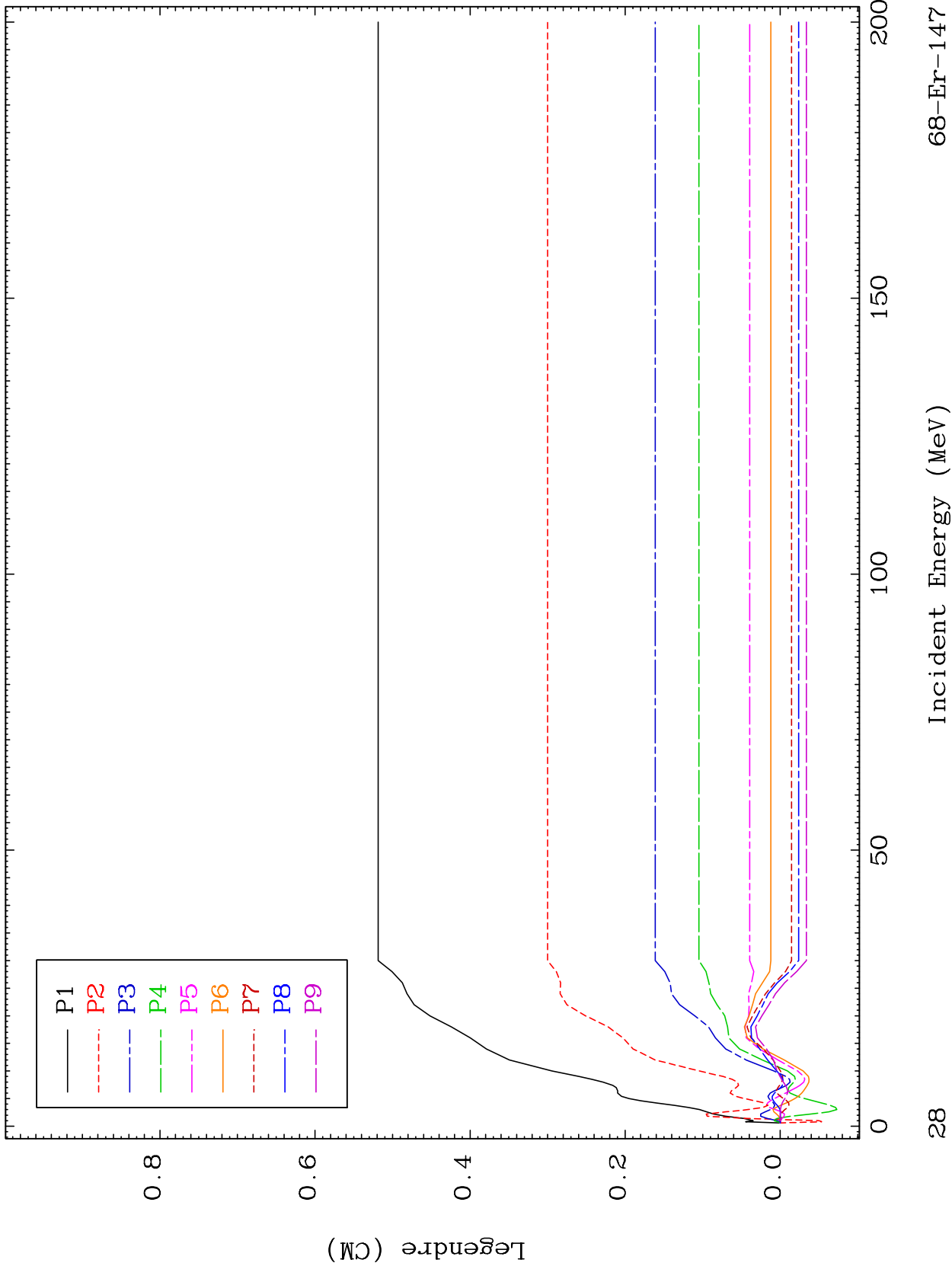




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MT= 54 (n,n') Level  
Legendre Coefficients

68-Er-147



68-Er-147

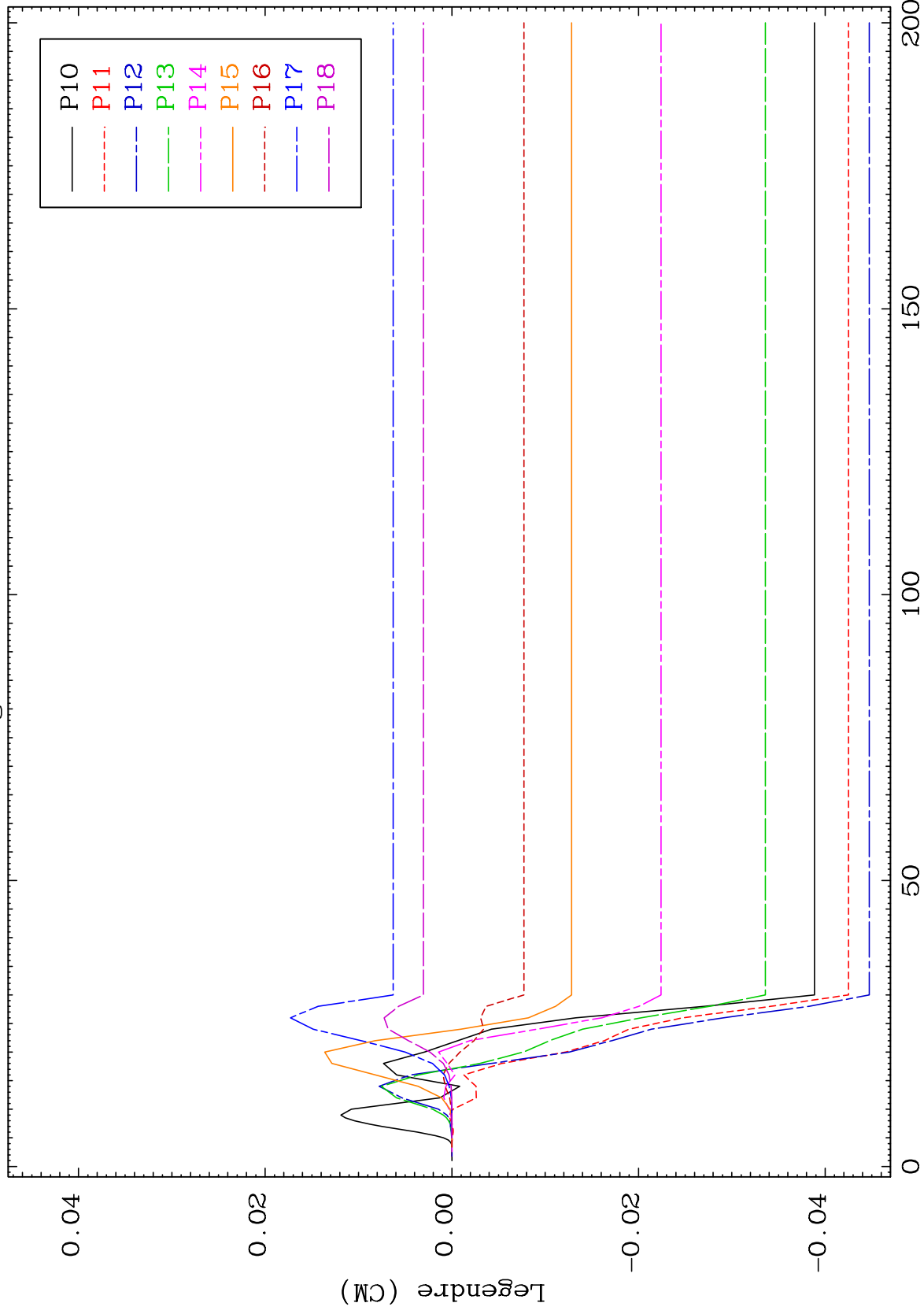
Incident Energy (MeV)

28

MAT 6780

MT= 54 (n,n') Level  
Legendre Coefficients

68-Er-147



29

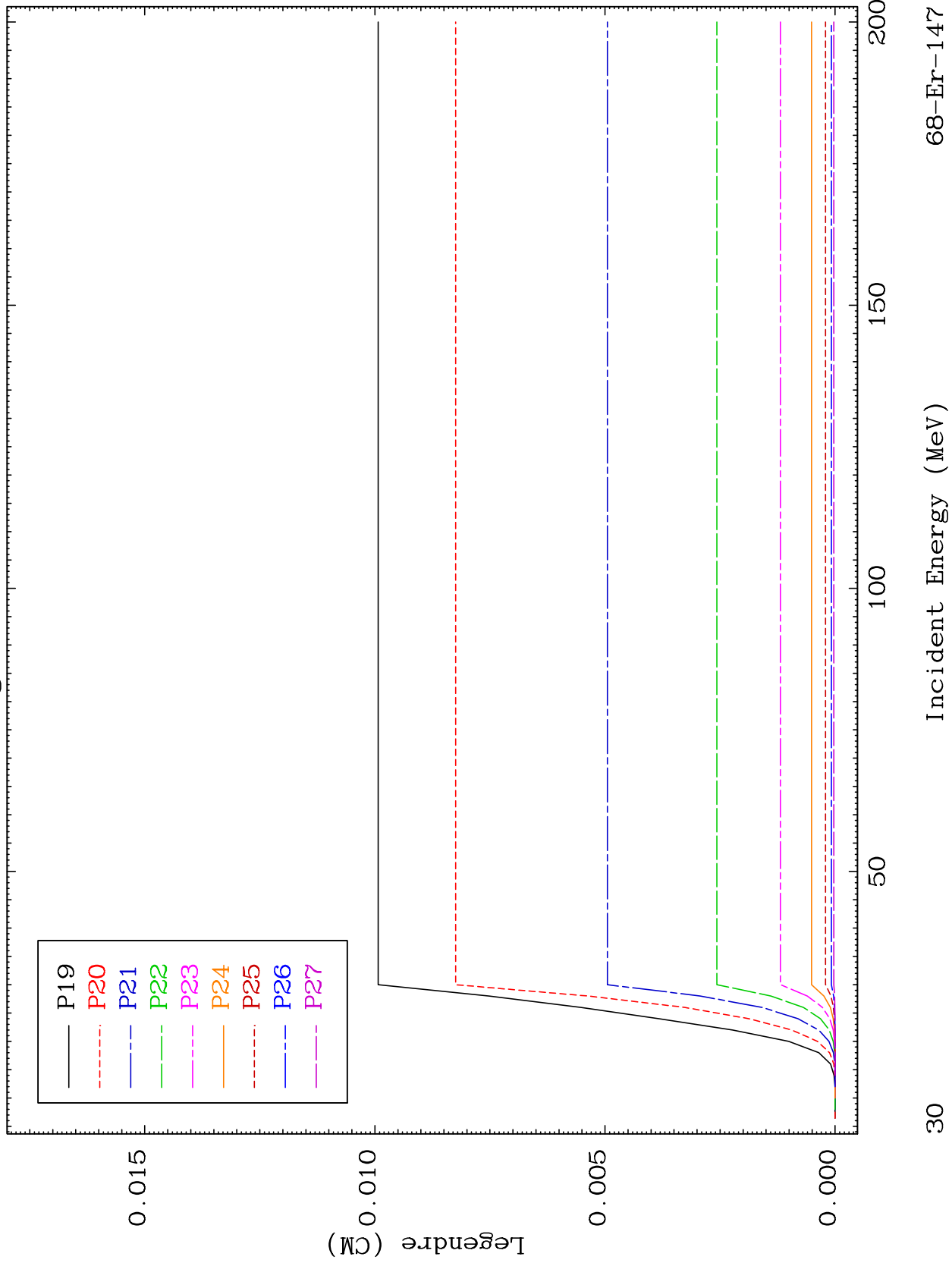
Incident Energy (MeV)

68-Er-147

MAT 6780

MT= 54 (n,n') Level  
Legendre Coefficients

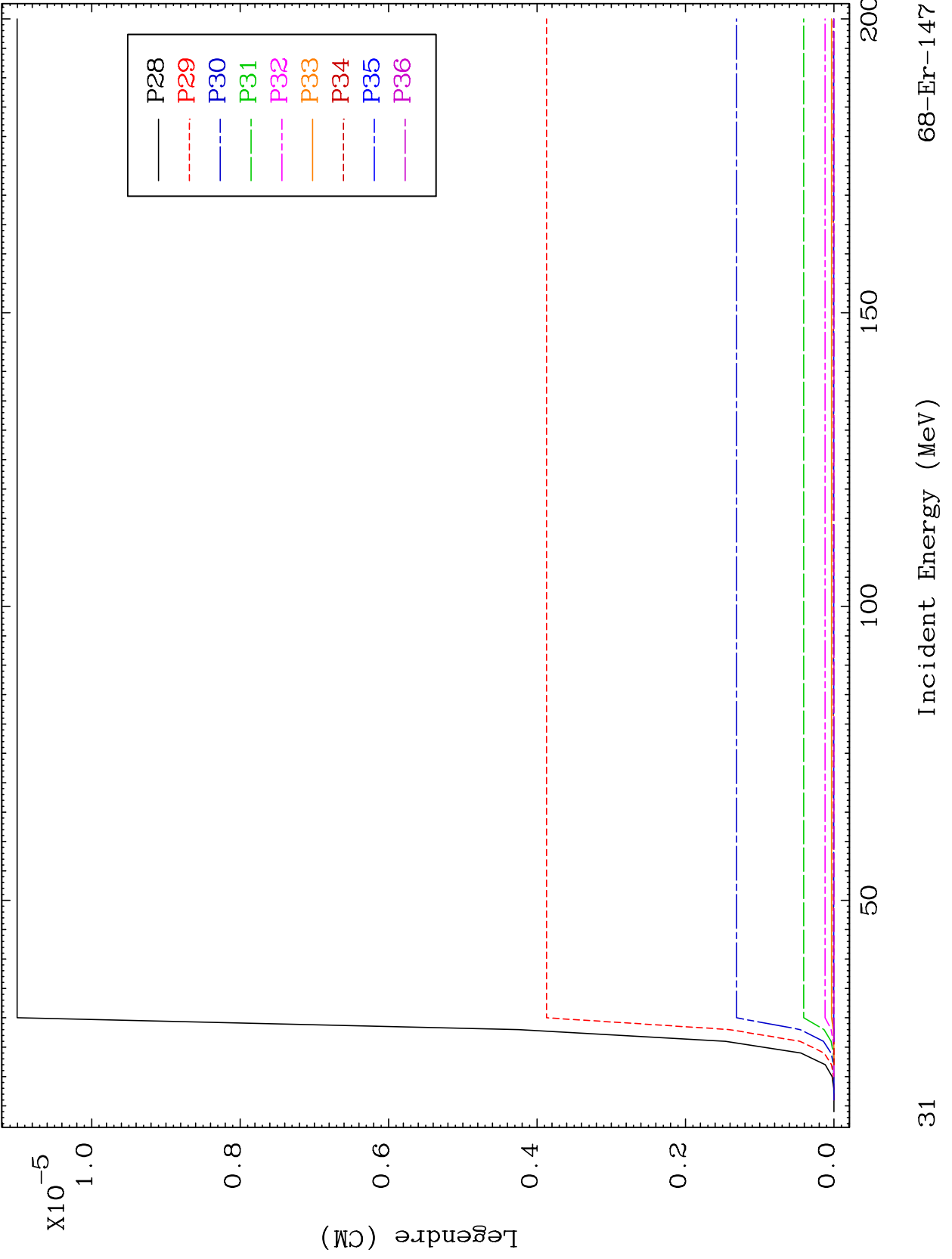
68-Er-147



68-Er-147

Incident Energy (MeV)

30

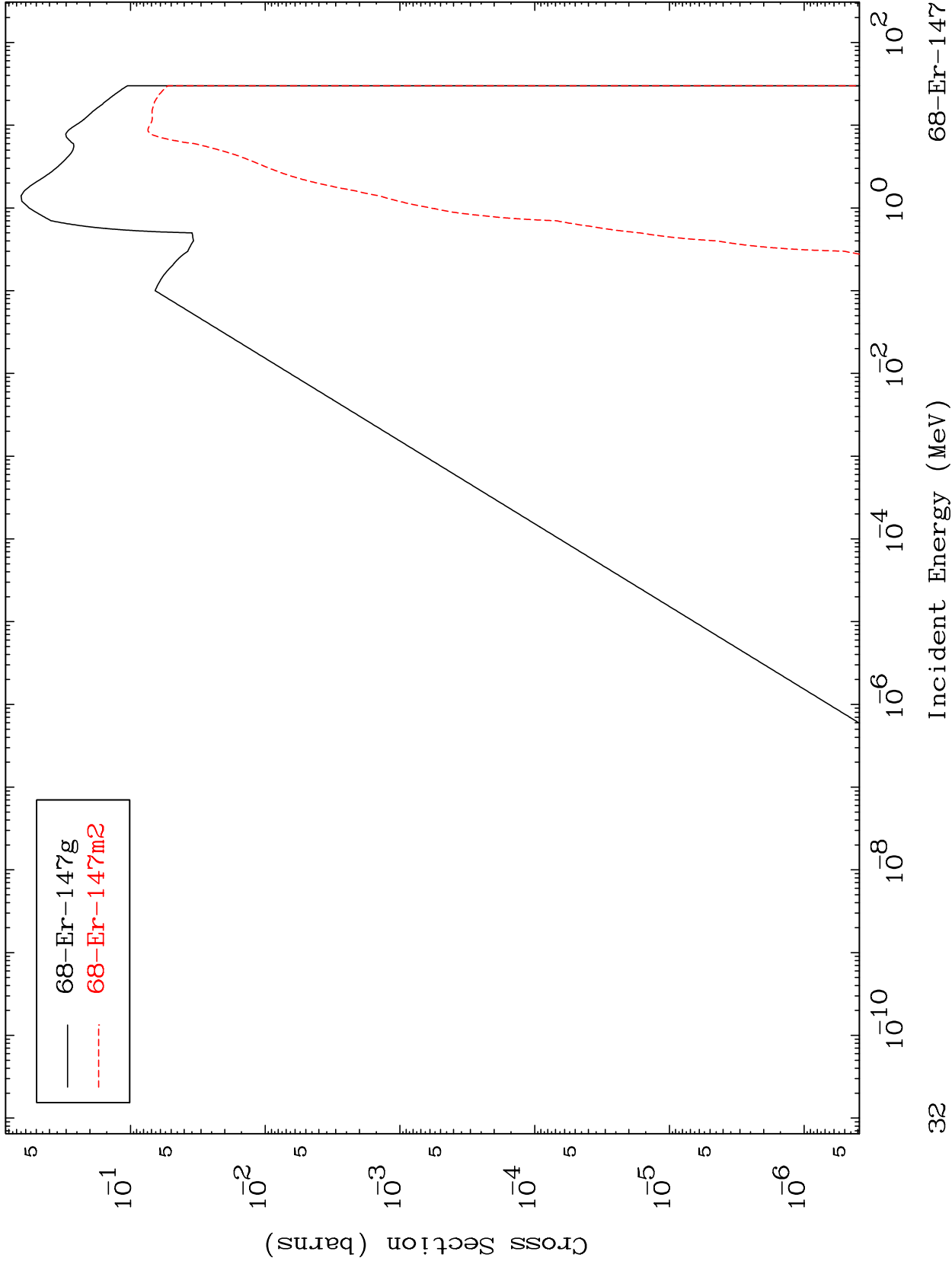




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Inelastic  
Radionuclide Production Cross Section

68-Er-147



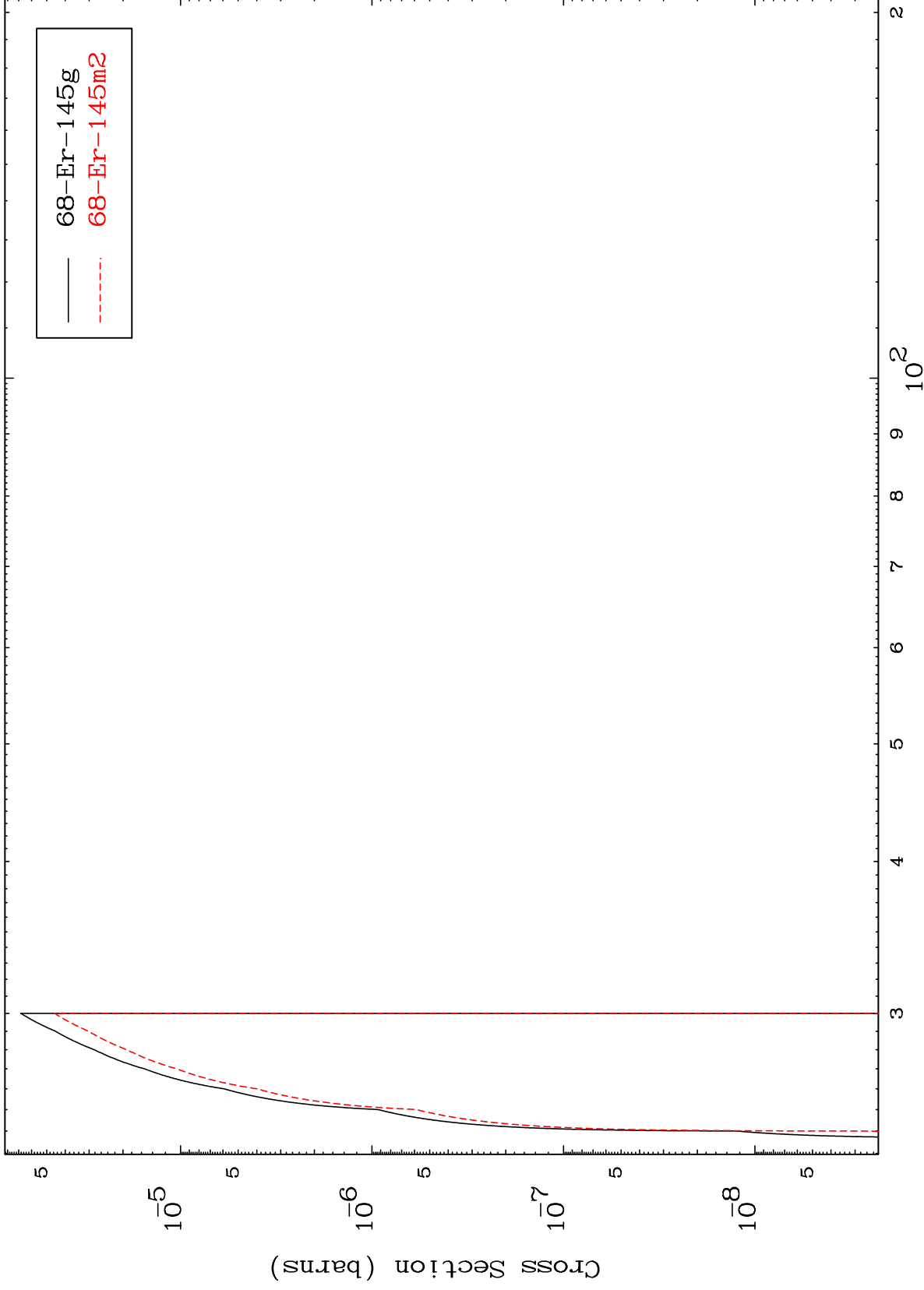
68-Er-147

MAT 6780

(n,3n)

68-Er-147

Radionuclide Production Cross Section



33

Incident Energy (MeV)

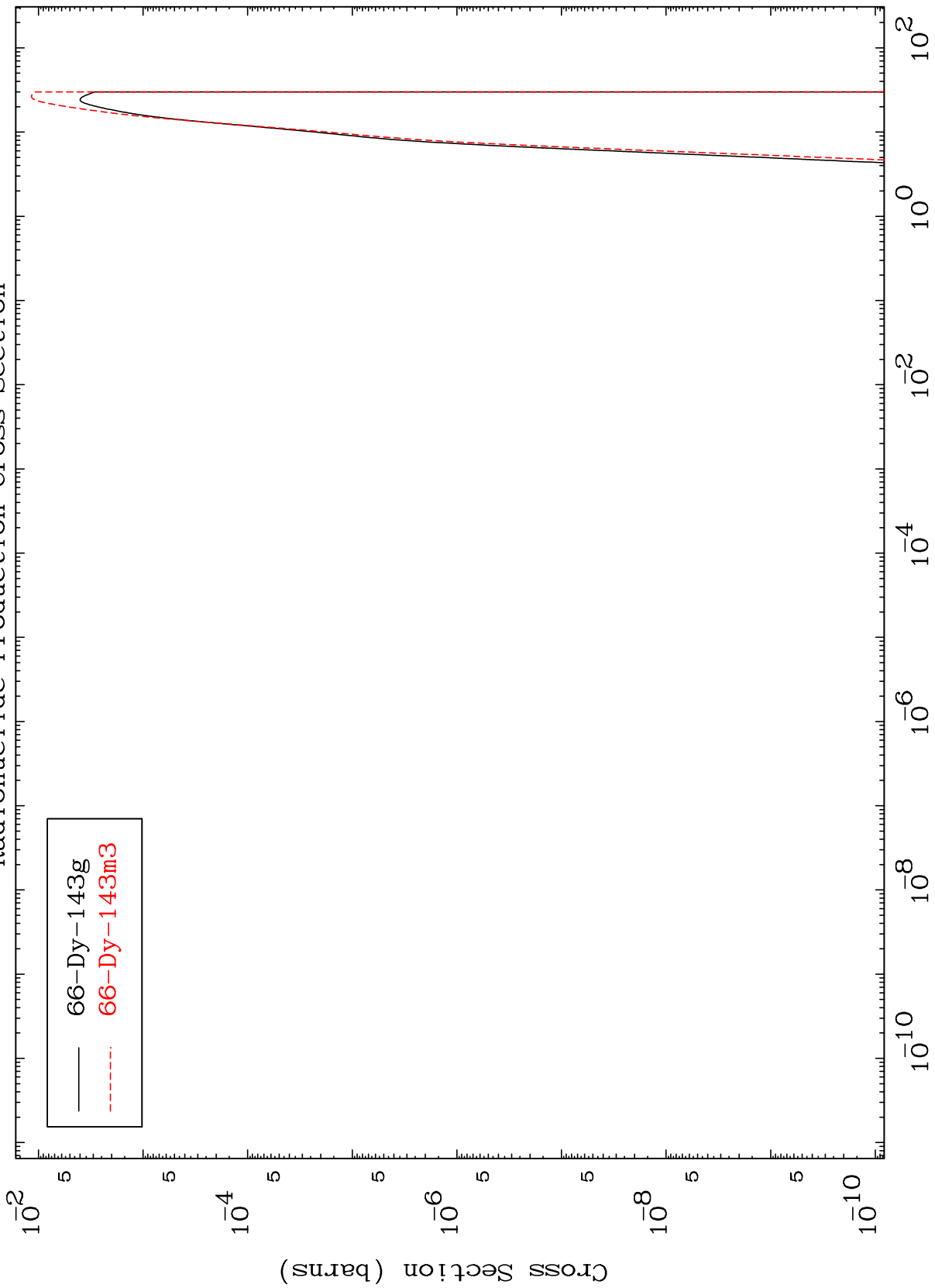
68-Er-147

MAT 6780

$(n, n') \alpha$

68-Er-147

Radionuclide Production Cross Section

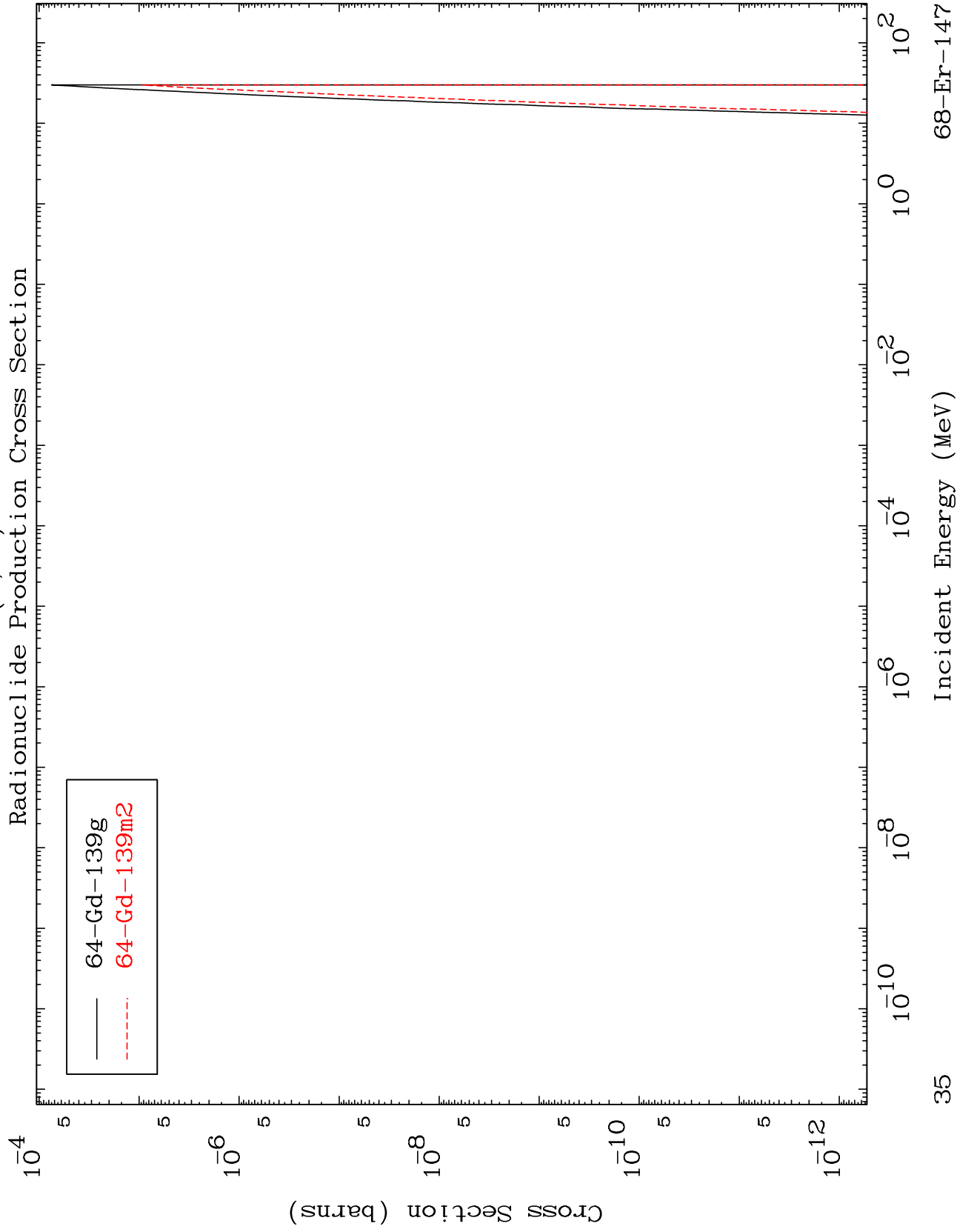


MAT 6780

(n,n') 2 $\alpha$

68-Er-147

Radionuclide Production Cross Section



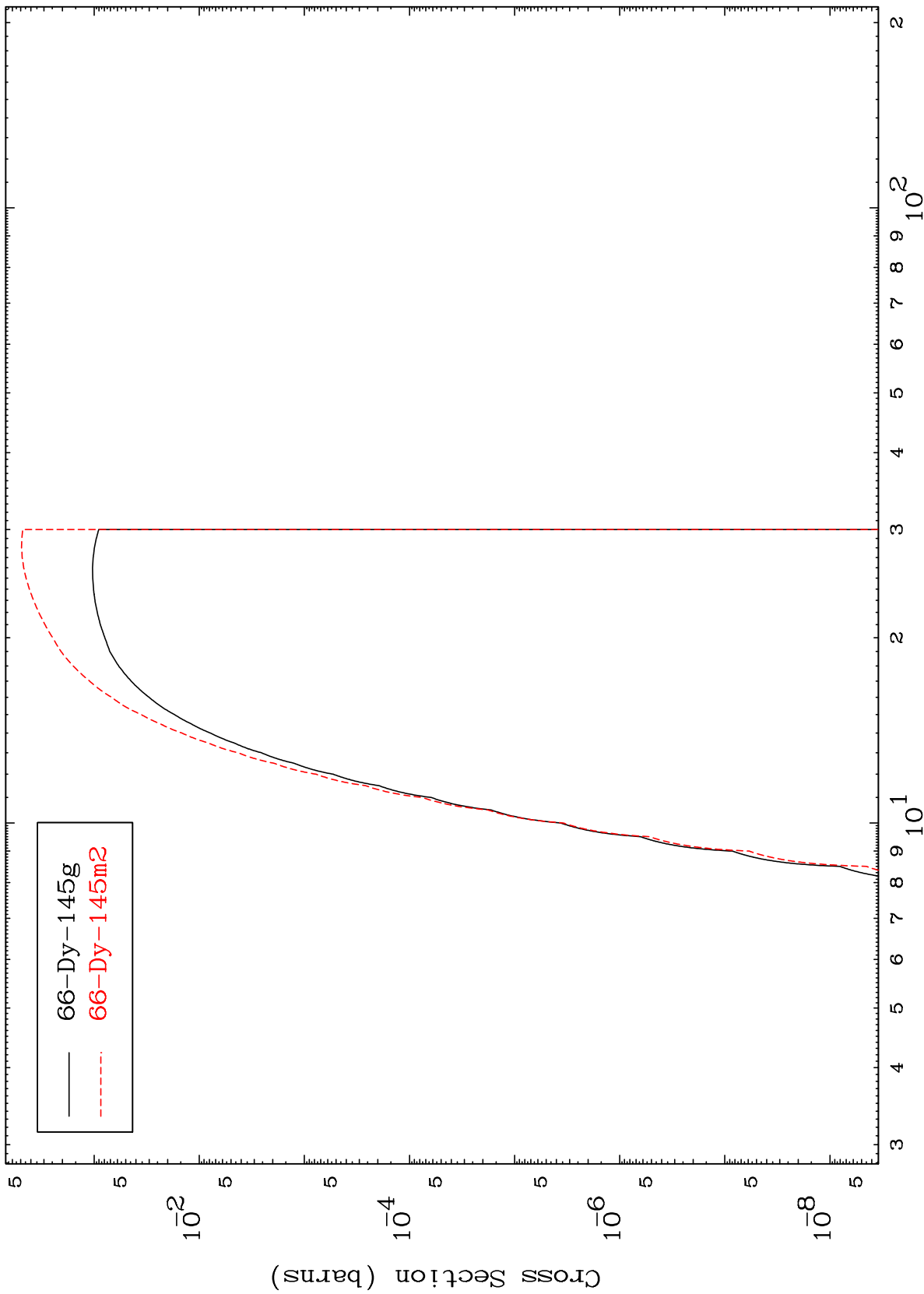
35

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(n,2n) p

68-Er-147

Radionuclide Production Cross Section



Incident Energy (MeV)

68-Er-147

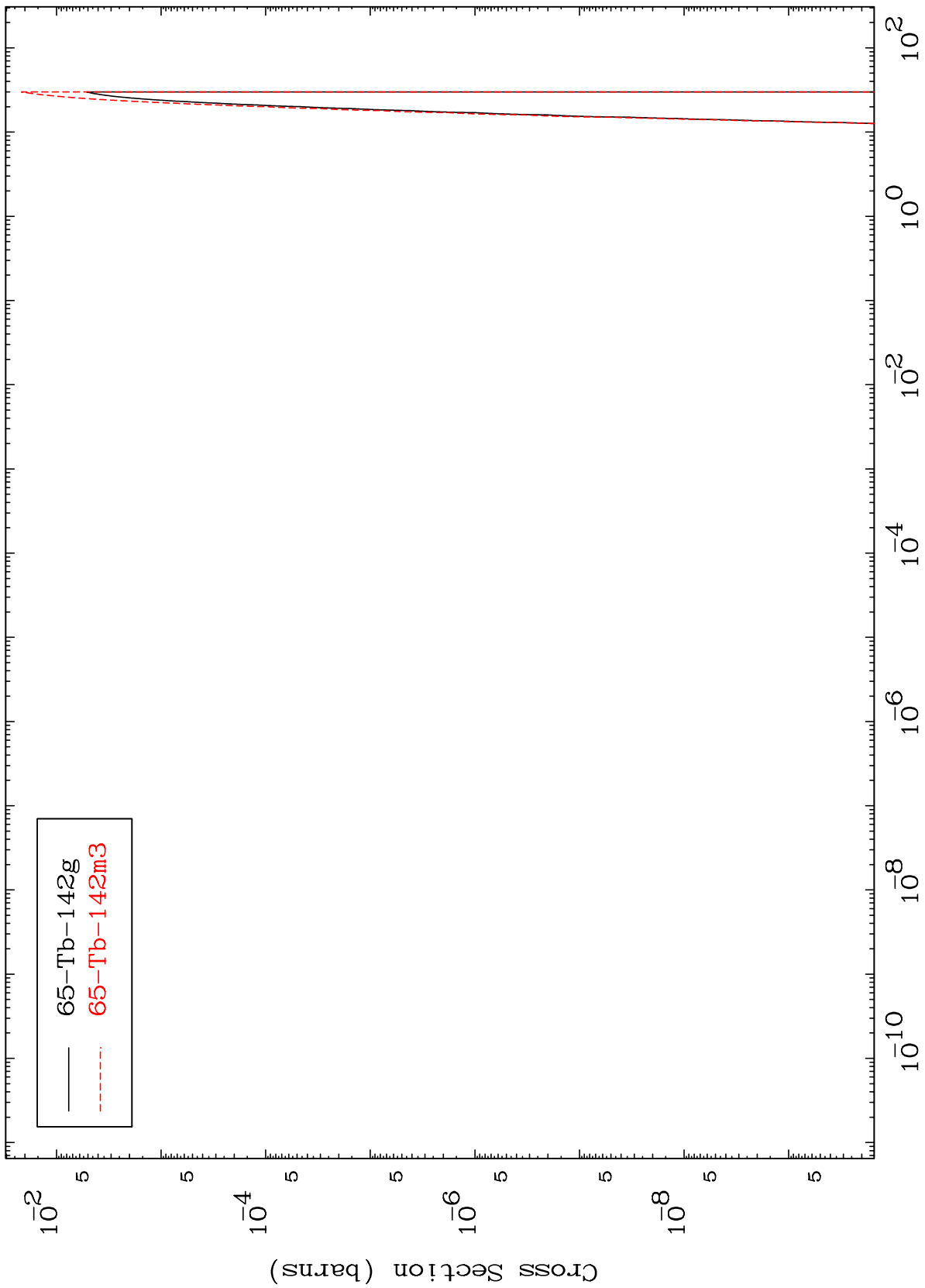
36

MAT 6780

(n,n') p  $\alpha$

68-Er-147

Radionuclide Production Cross Section

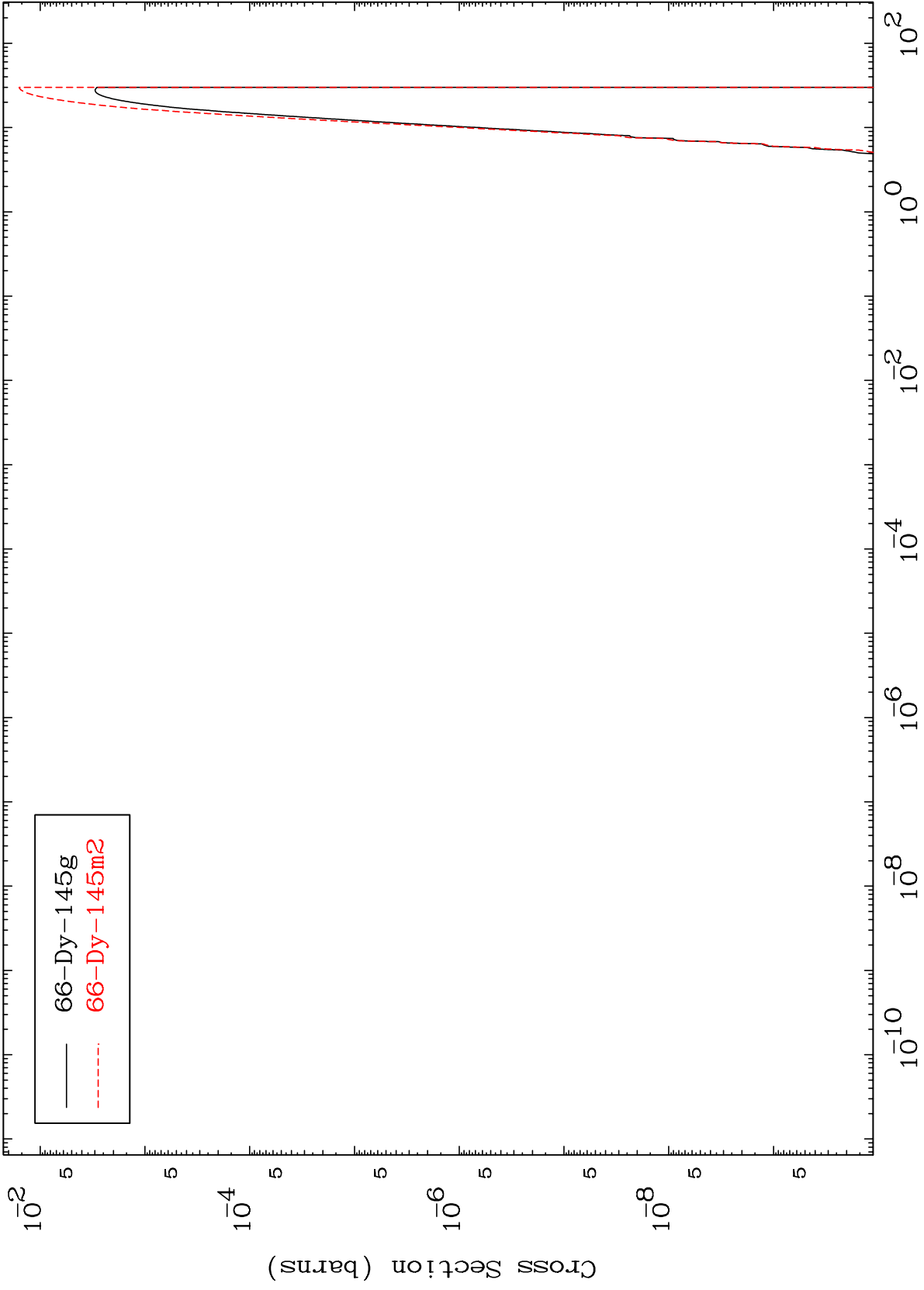


MAT 6780

(n,He-3)

68-Er-147

Radionuclide Production Cross Section



38

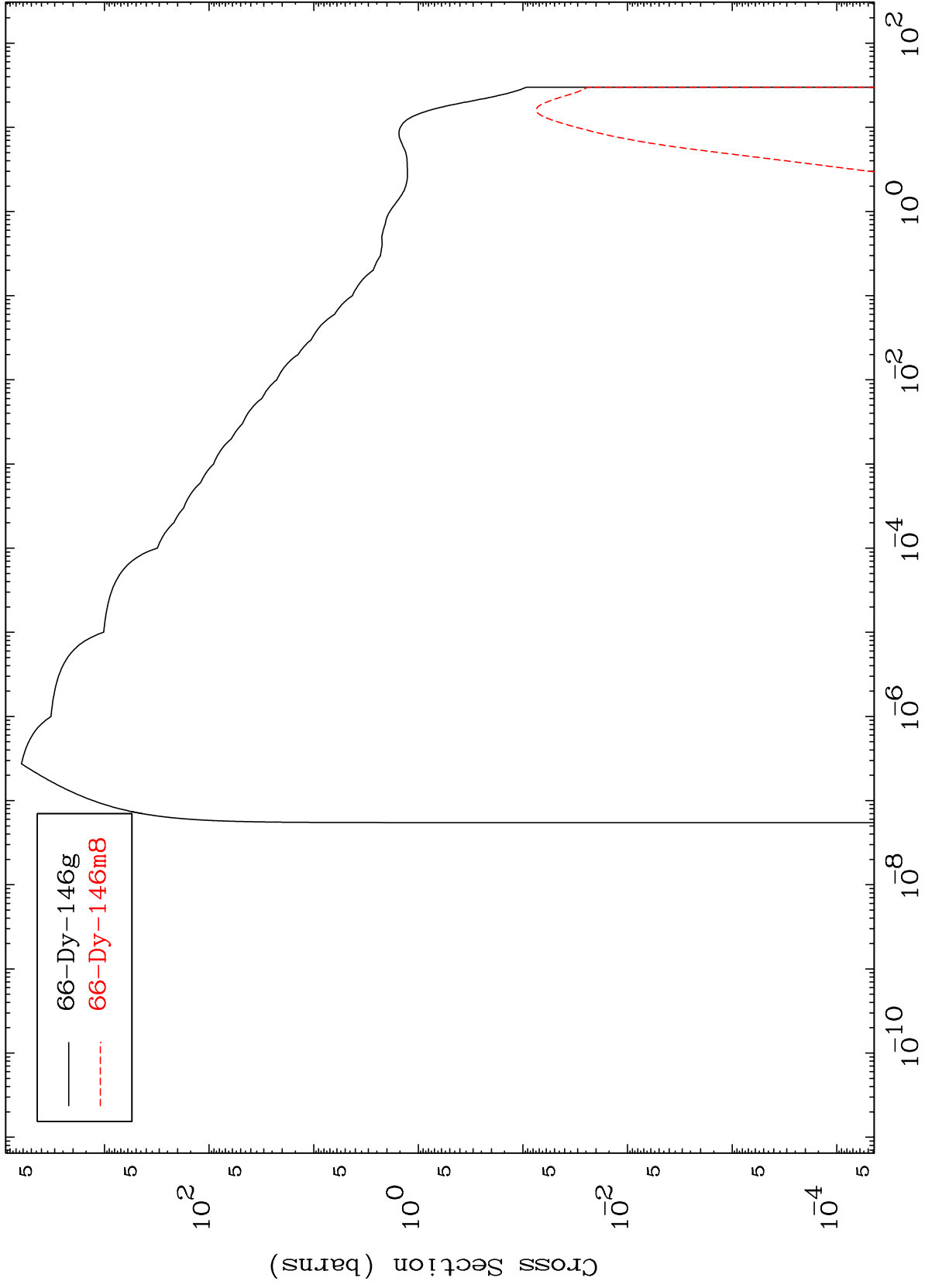
Incident Energy (MeV)

68-Er-147

MAT 6780

68-Er-147

(n,2p)  
Radionuclide Production Cross Section



66-Dy-146g  
66-Dy-146m8

68-Er-147

Incident Energy (MeV)

39

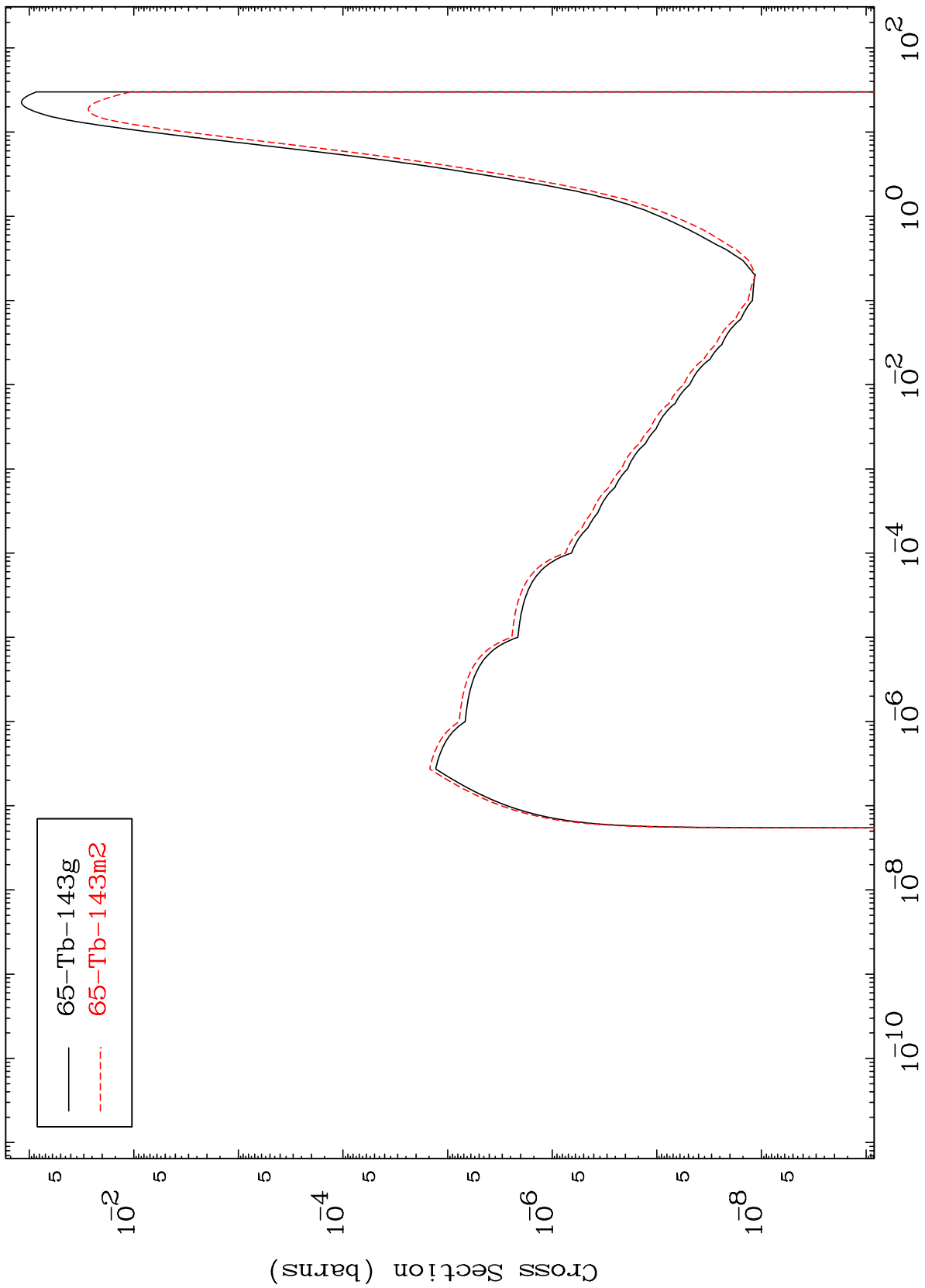


MAT 6780

(n,p)  $\alpha$

68-Er-147

Radionuclide Production Cross Section



40

Incident Energy (MeV)

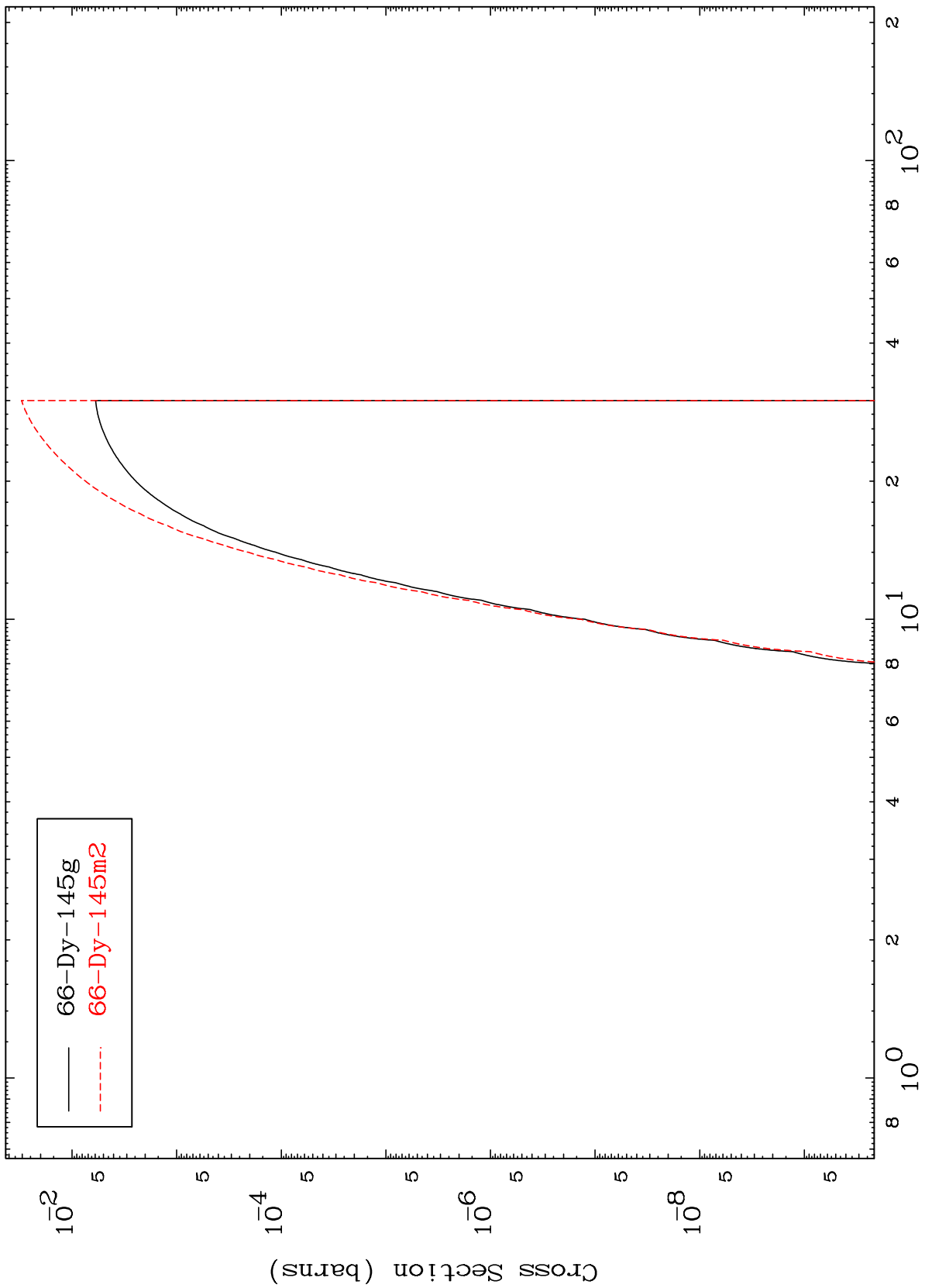
68-Er-147

MAT 6780

(n,p) d

68-Er-147

Radionuclide Production Cross Section



— 66-Dy-145g  
- - - 66-Dy-145m2

41

Incident Energy (MeV)

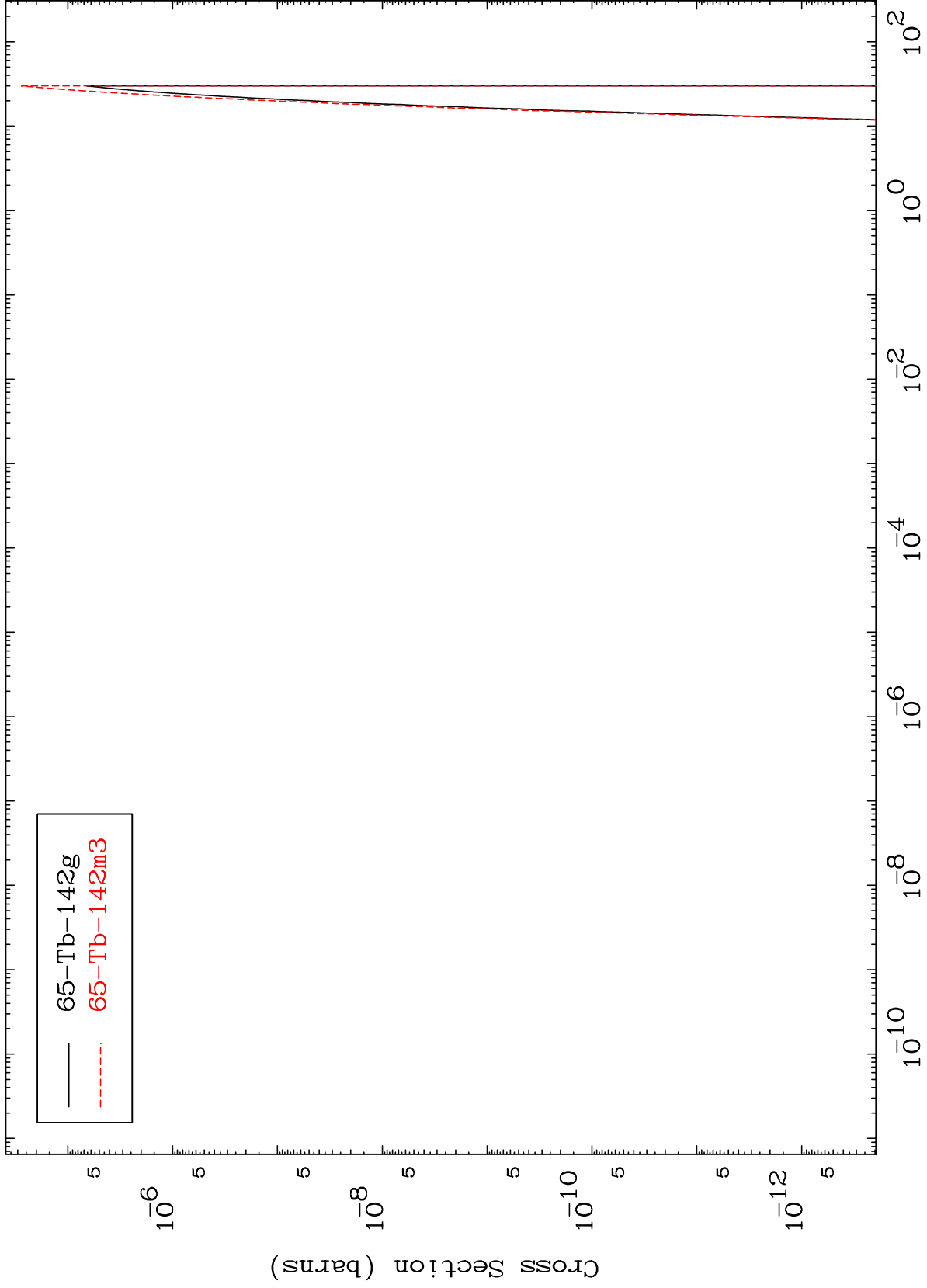
68-Er-147

MAT 6780

(n,d)  $\alpha$

68-Er-147

Radionuclide Production Cross Section



42

Incident Energy (MeV)

68-Er-147