

Program Complot
(Version 2018-1)

by

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Press Mouse Button to Start

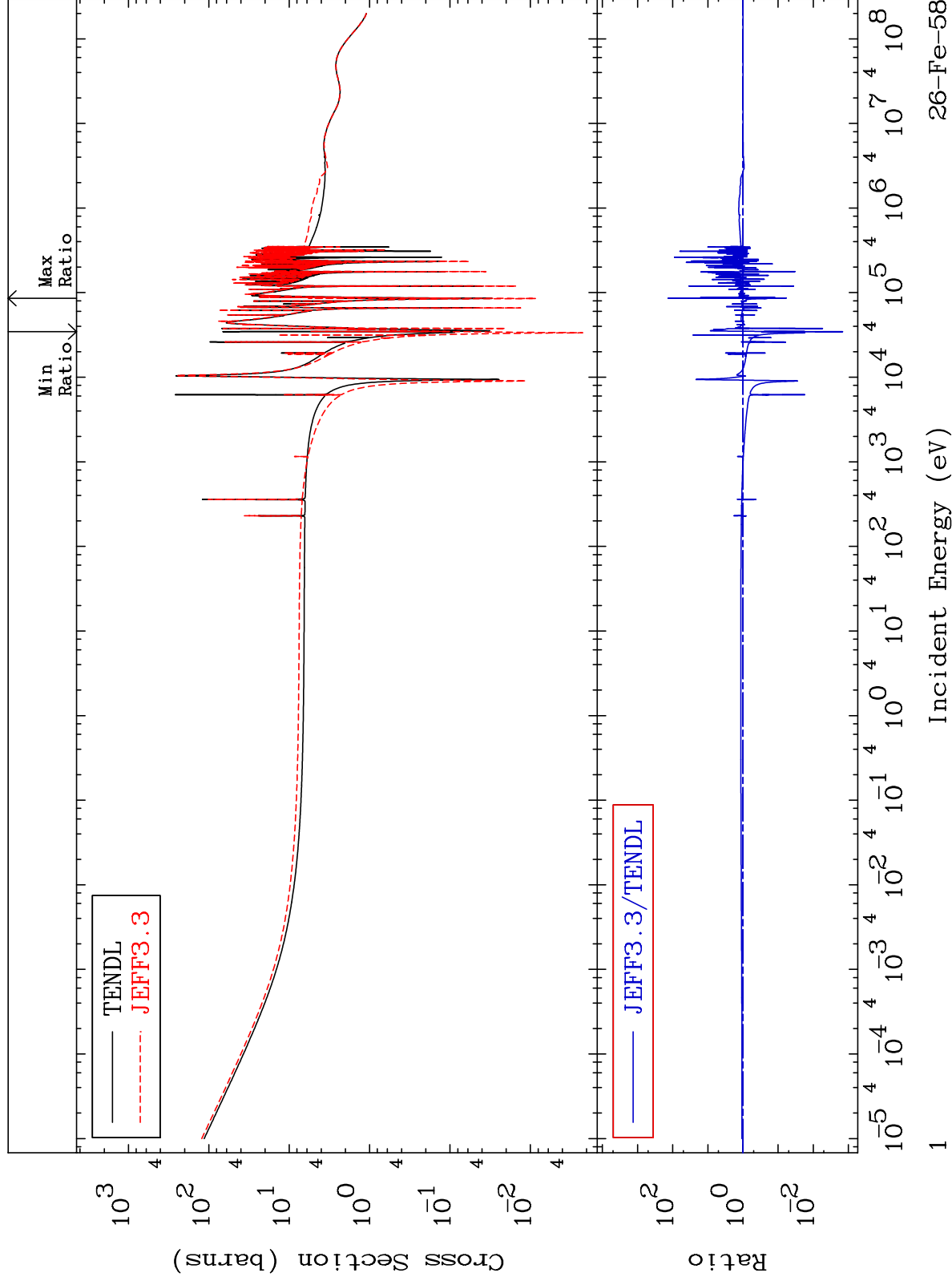
MAT 2637

Total

²⁶Fe-58

Cross Section

-99.86 To 9999. %



Incident Energy (eV)

²⁶Fe-58

1

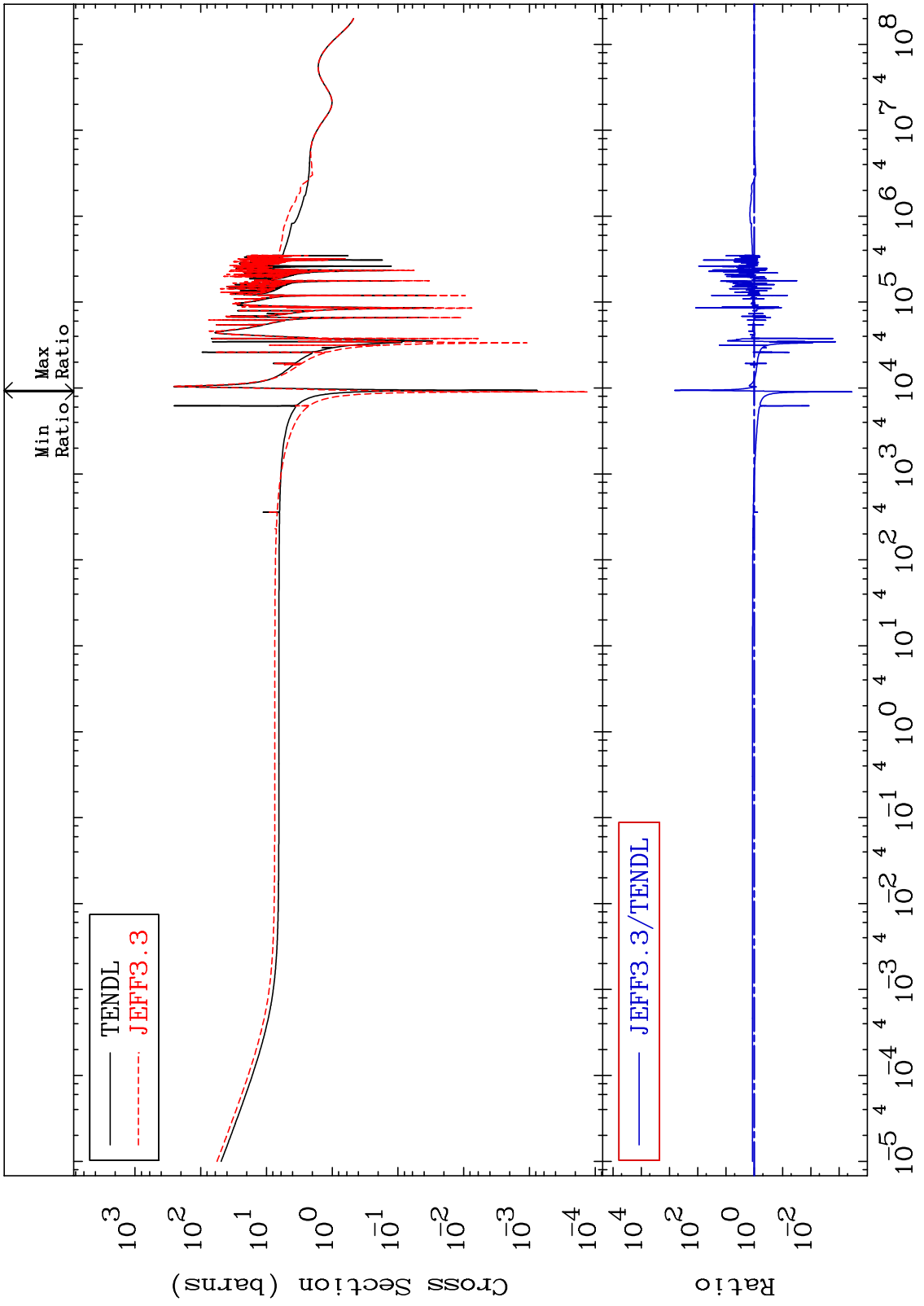
MAT 2637

Elastic

²⁶Fe-58

Cross Section

-99.97 To 9999. %

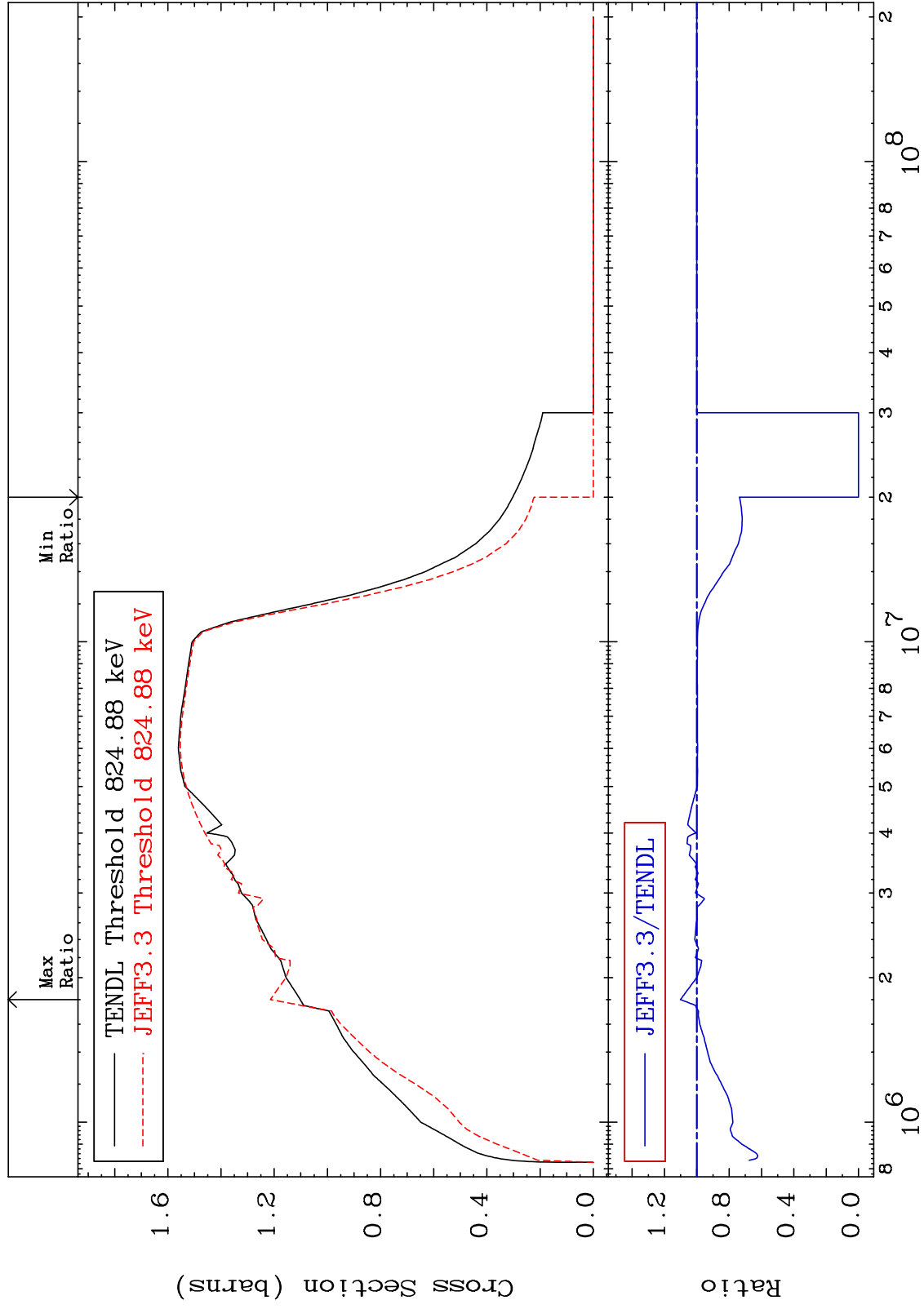


MAT 2637

Inelastic
Cross Section

²⁶Fe-58

-100.0 To 10.12 %



Incident Energy (eV)

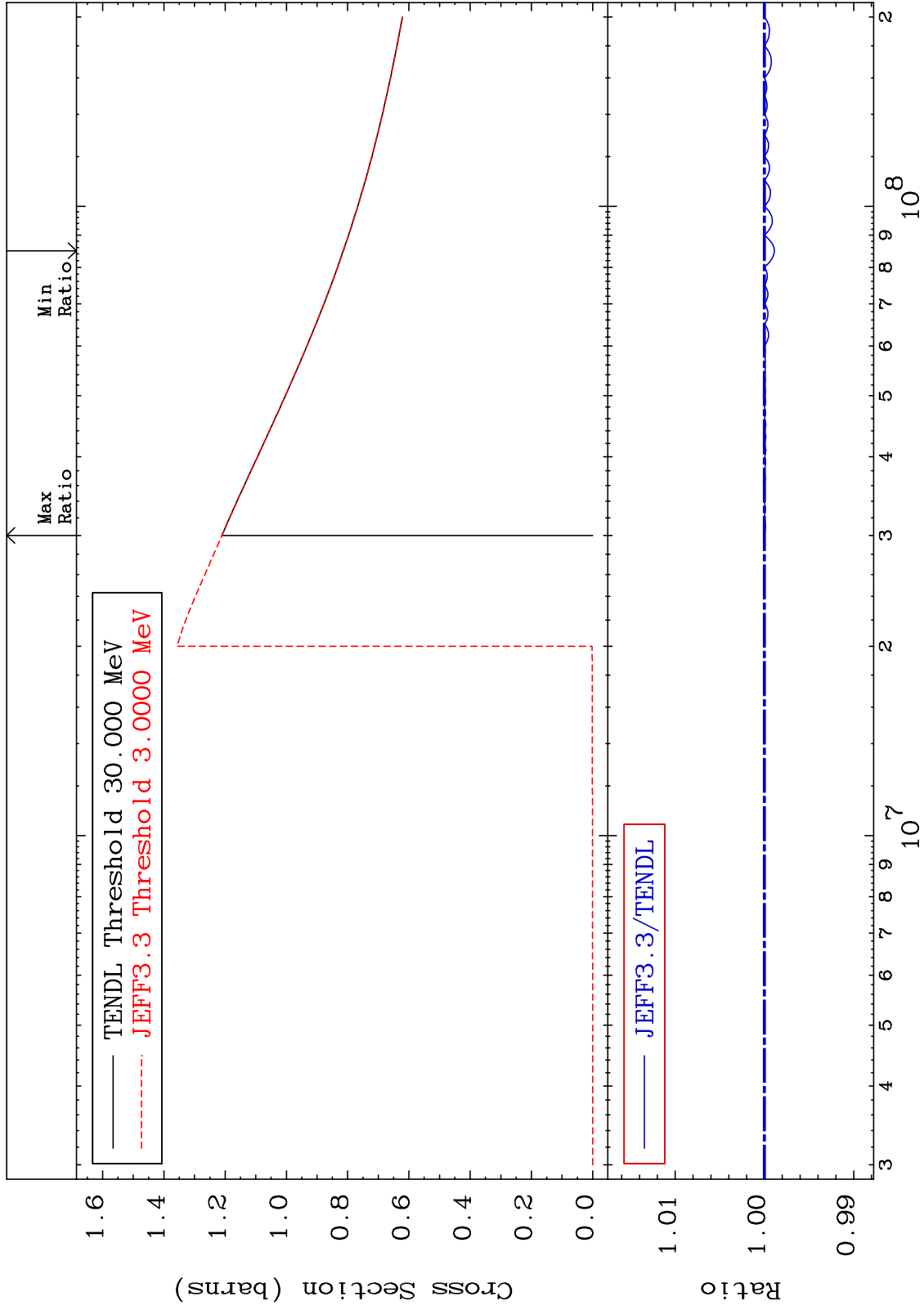
²⁶Fe-58

3

MAT 2637

(n, remainder)
Cross Section

26-Fe-58
-0.111 To 0.004 %



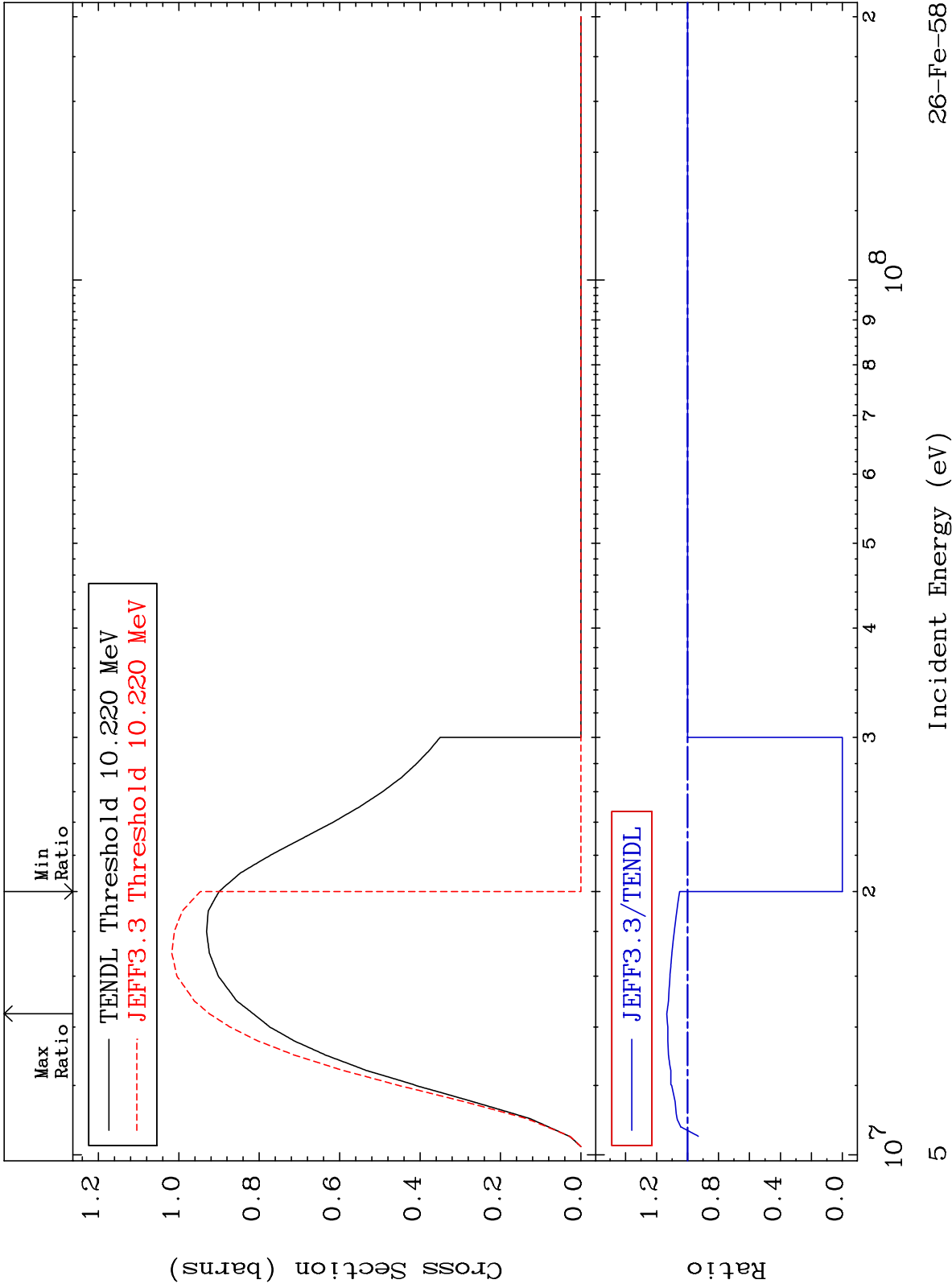
MAT 2637

(n,2n)

²⁶Fe-58

Cross Section

-100.0 To 13.39 %



Incident Energy (eV)

²⁶Fe-58

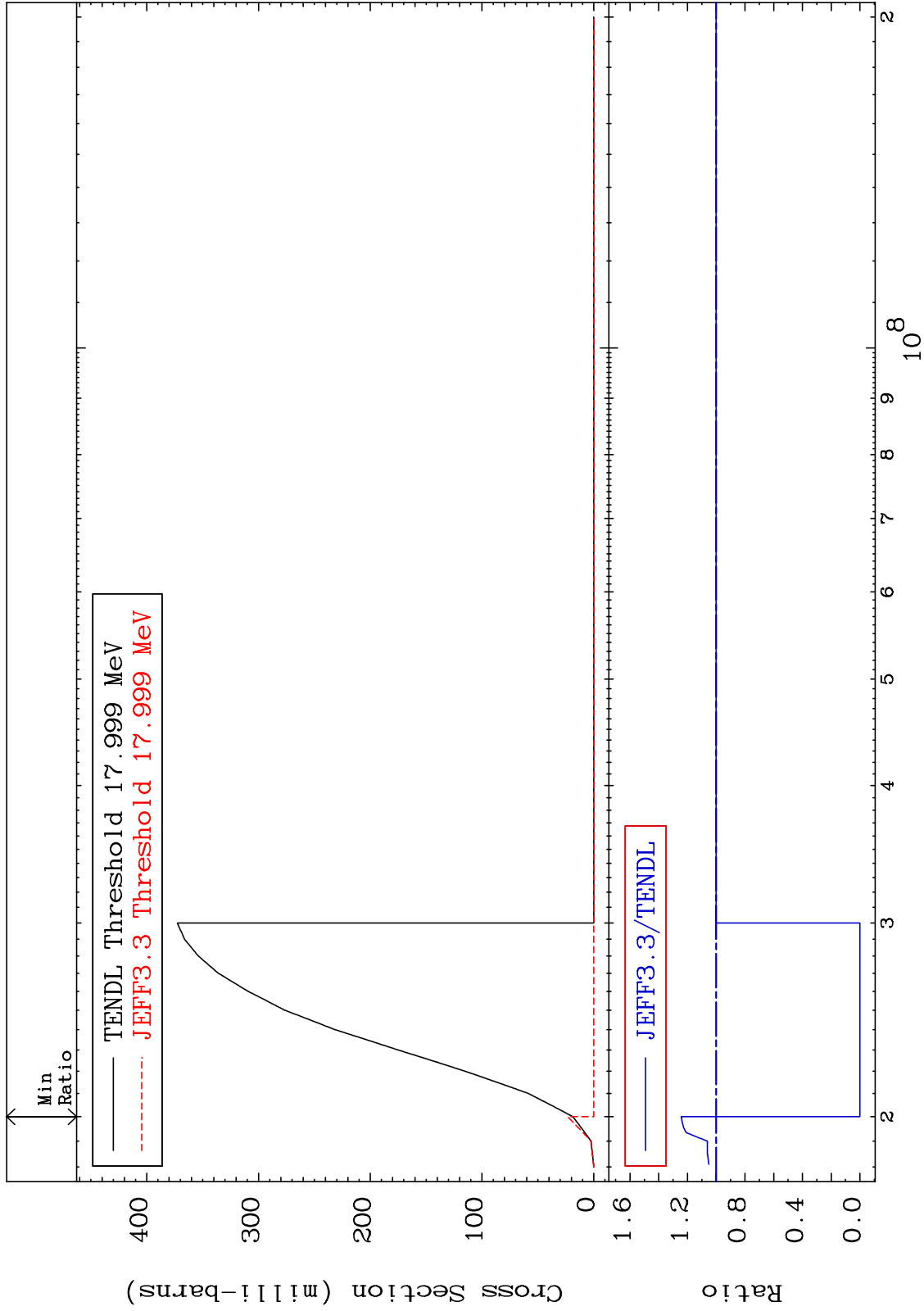
MAT 2637

(n, 3n)

²⁶Fe-58

Cross Section

-100.0 To 24.35 %



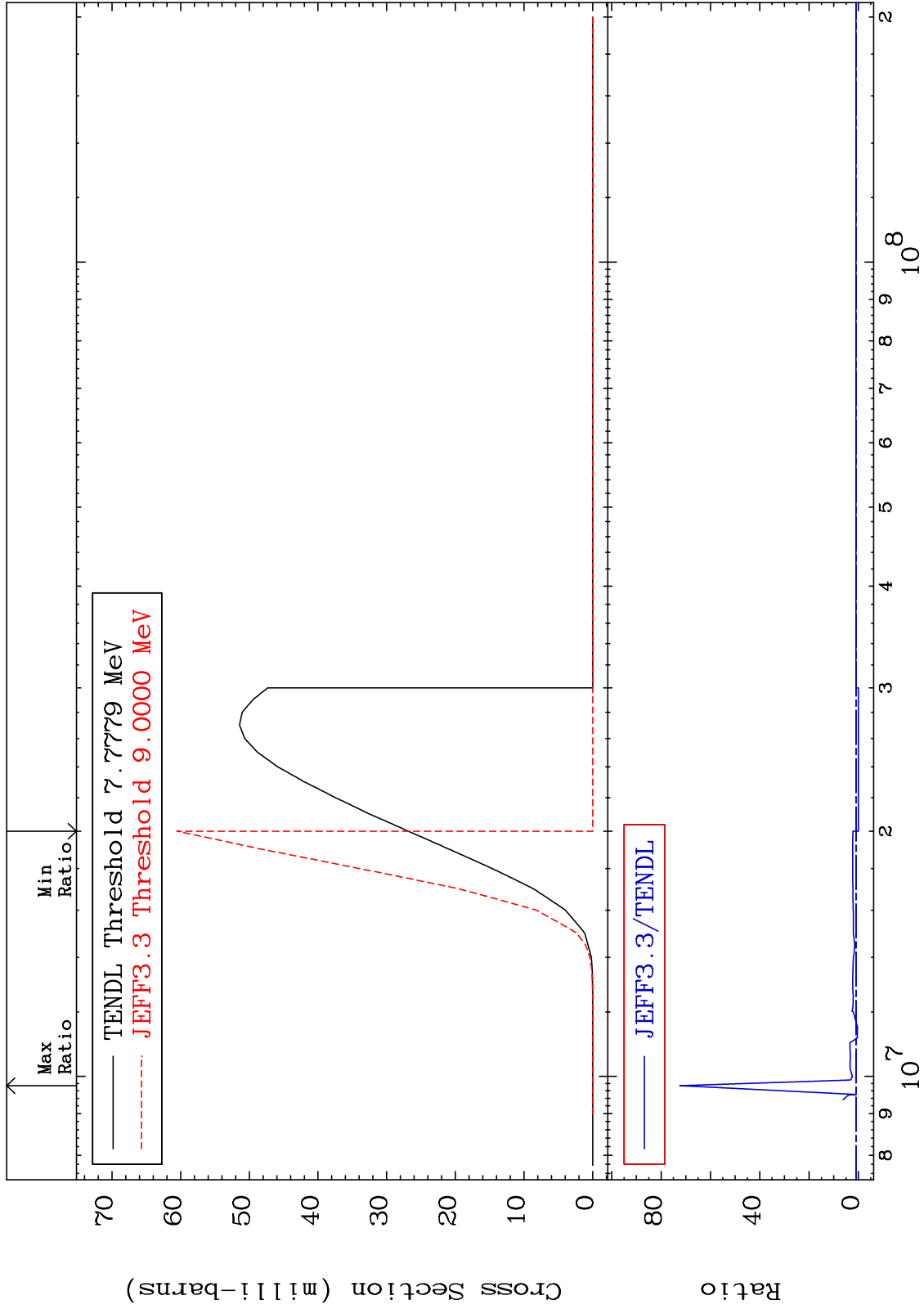
MAT 2637

(n, n') α

²⁶Fe-58

Cross Section

-100.0 To 7133. %



7

Incident Energy (eV)

²⁶Fe-58

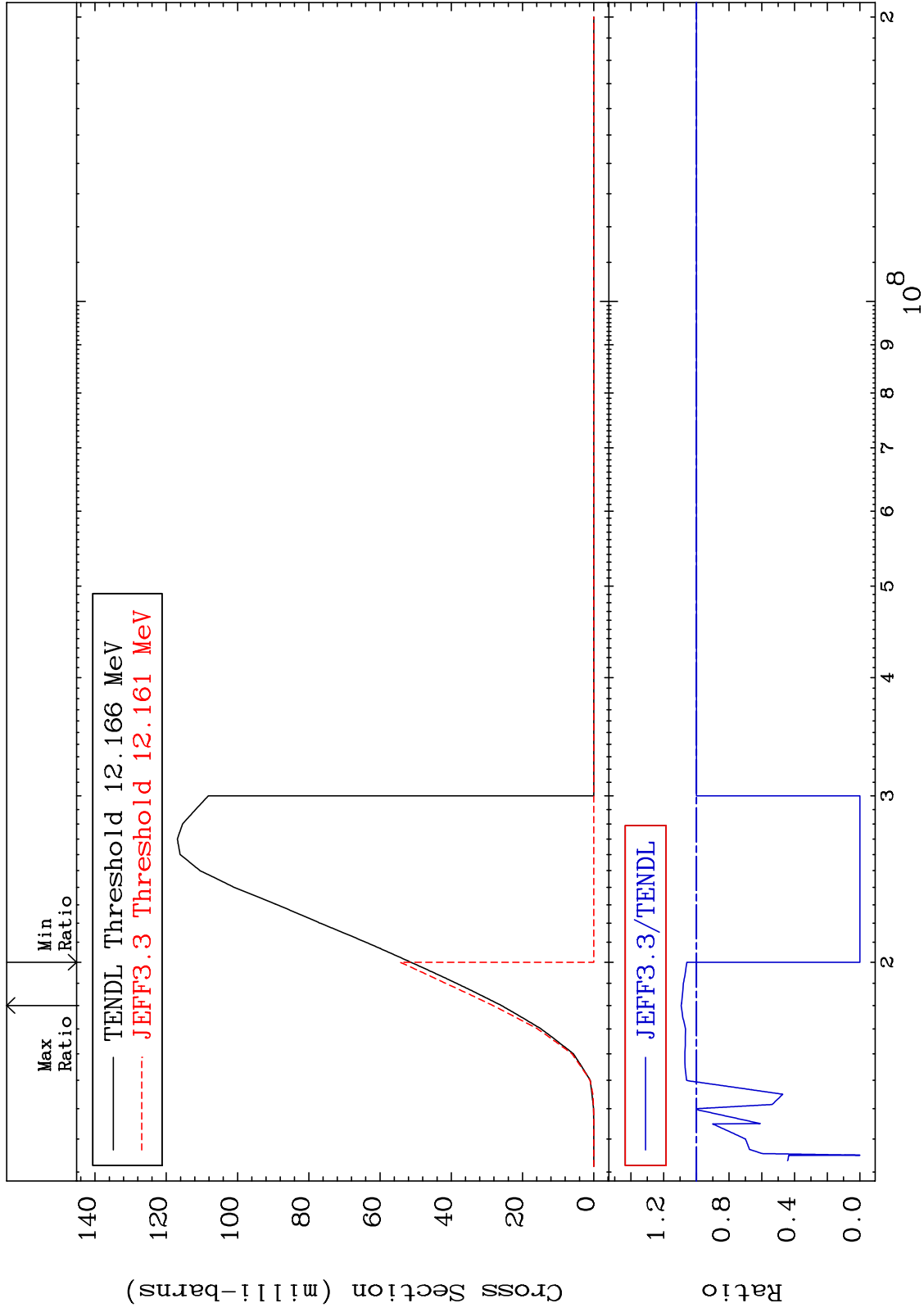
MAT 2637

(n, n') p

²⁶Fe-58

Cross Section

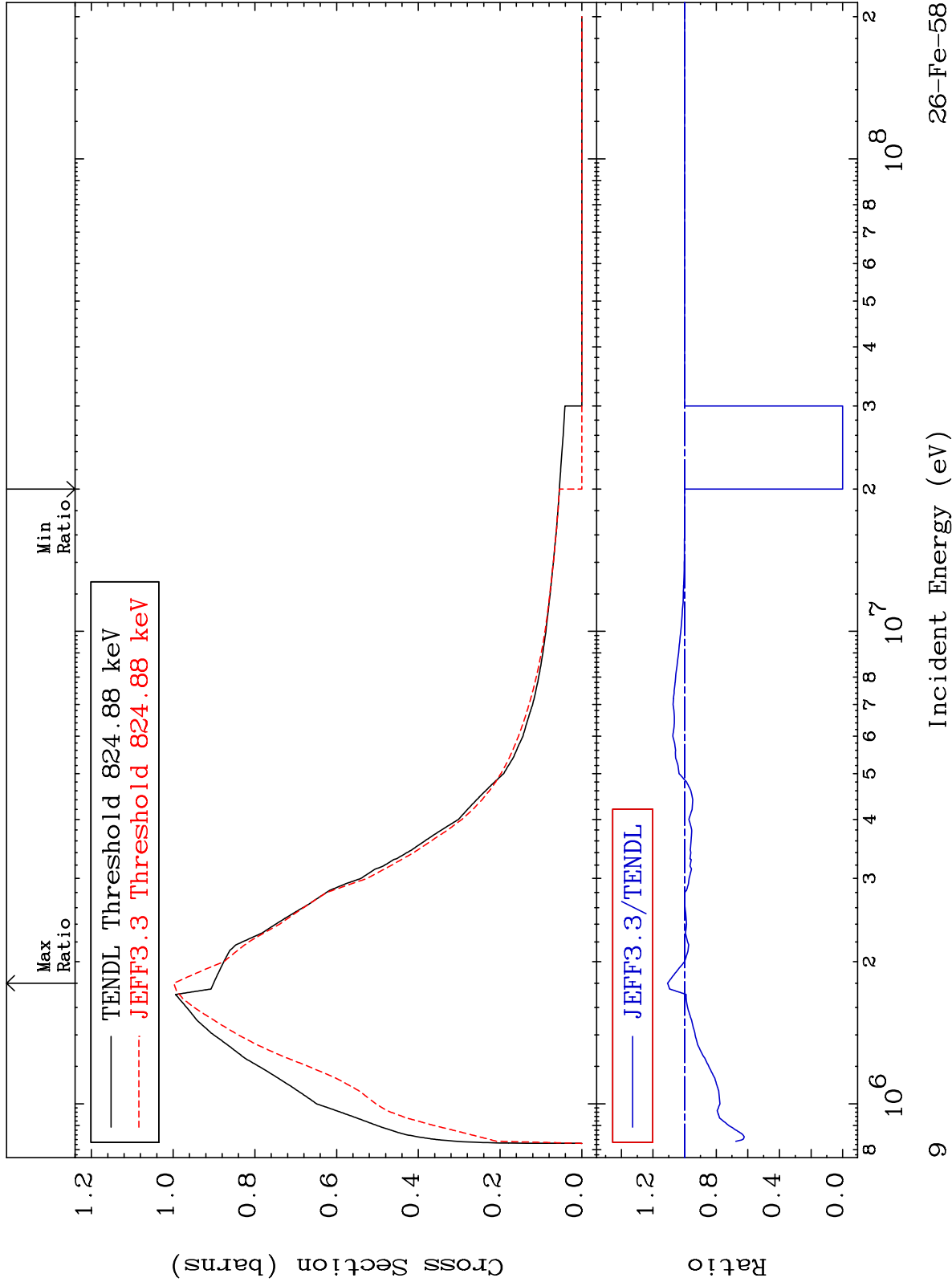
-100.0 To 9.277 %



MAT 2637

MT= 51 (n, n') Level
Cross Section

²⁶Fe-58
-100.0 To 10.79 %



²⁶Fe-58

Incident Energy (eV)

9

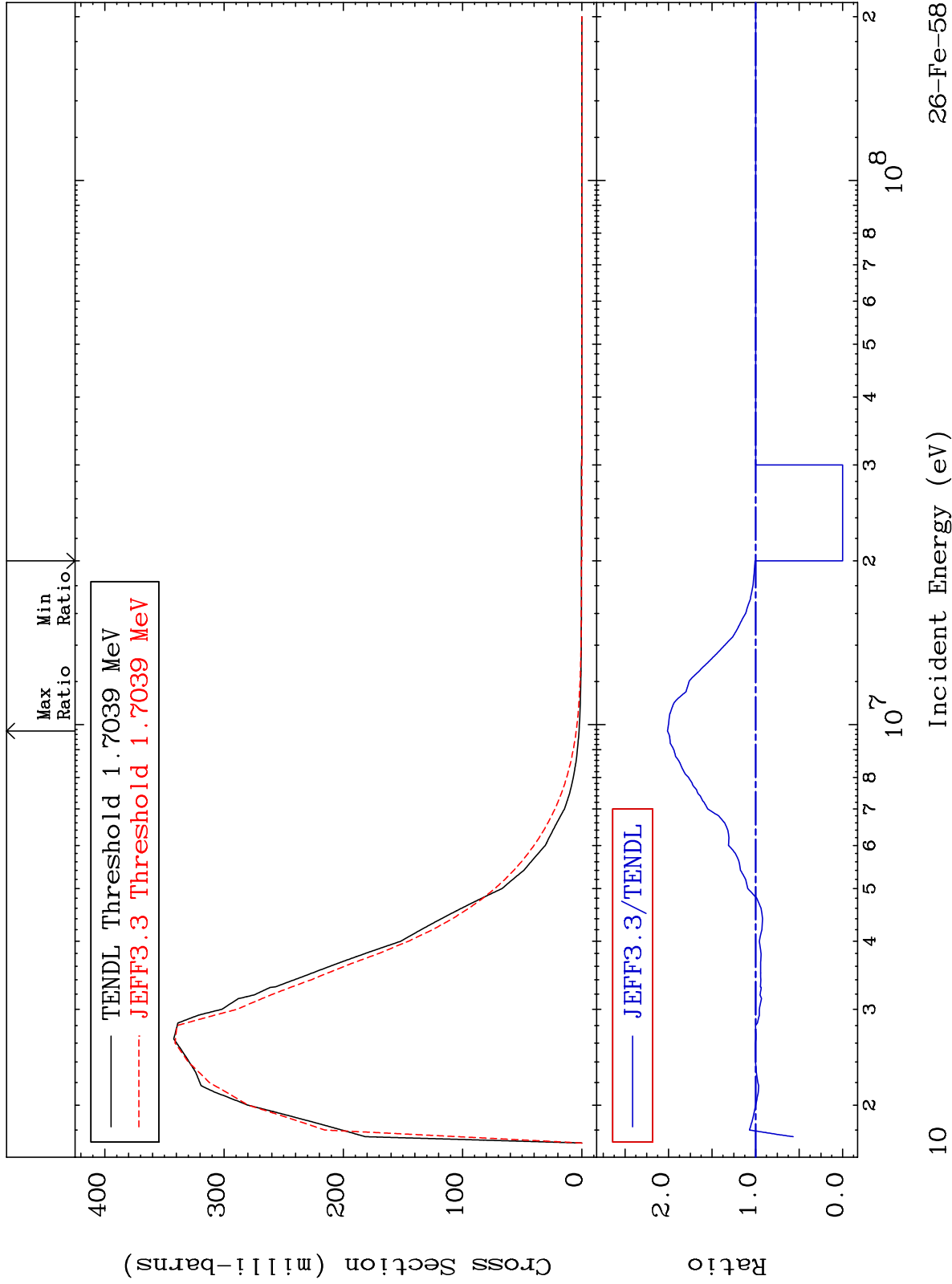
MAT 2637

MT= 52 (n,n') Level

²⁶Fe-58

-100.0 To 101.1 %

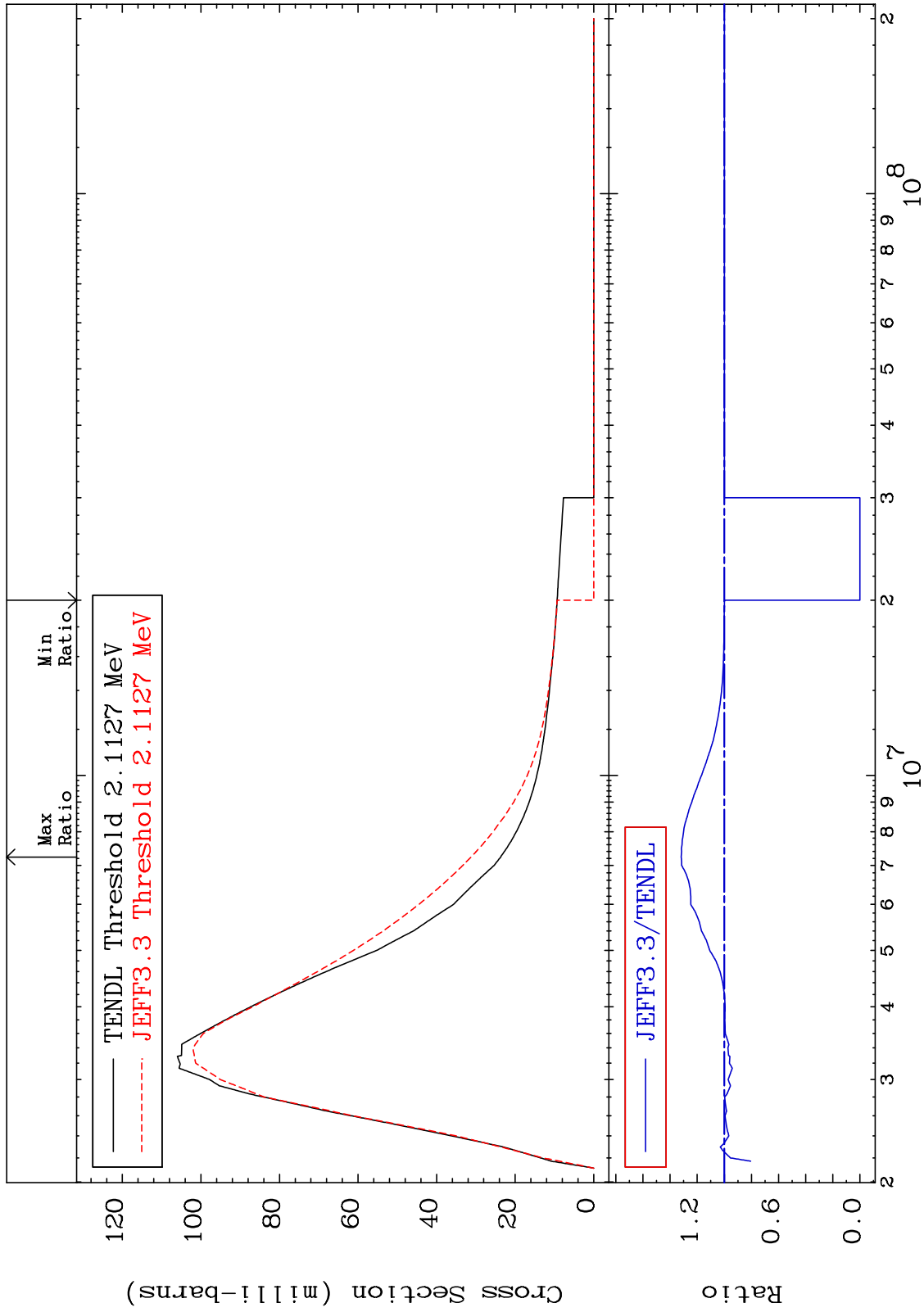
Cross Section



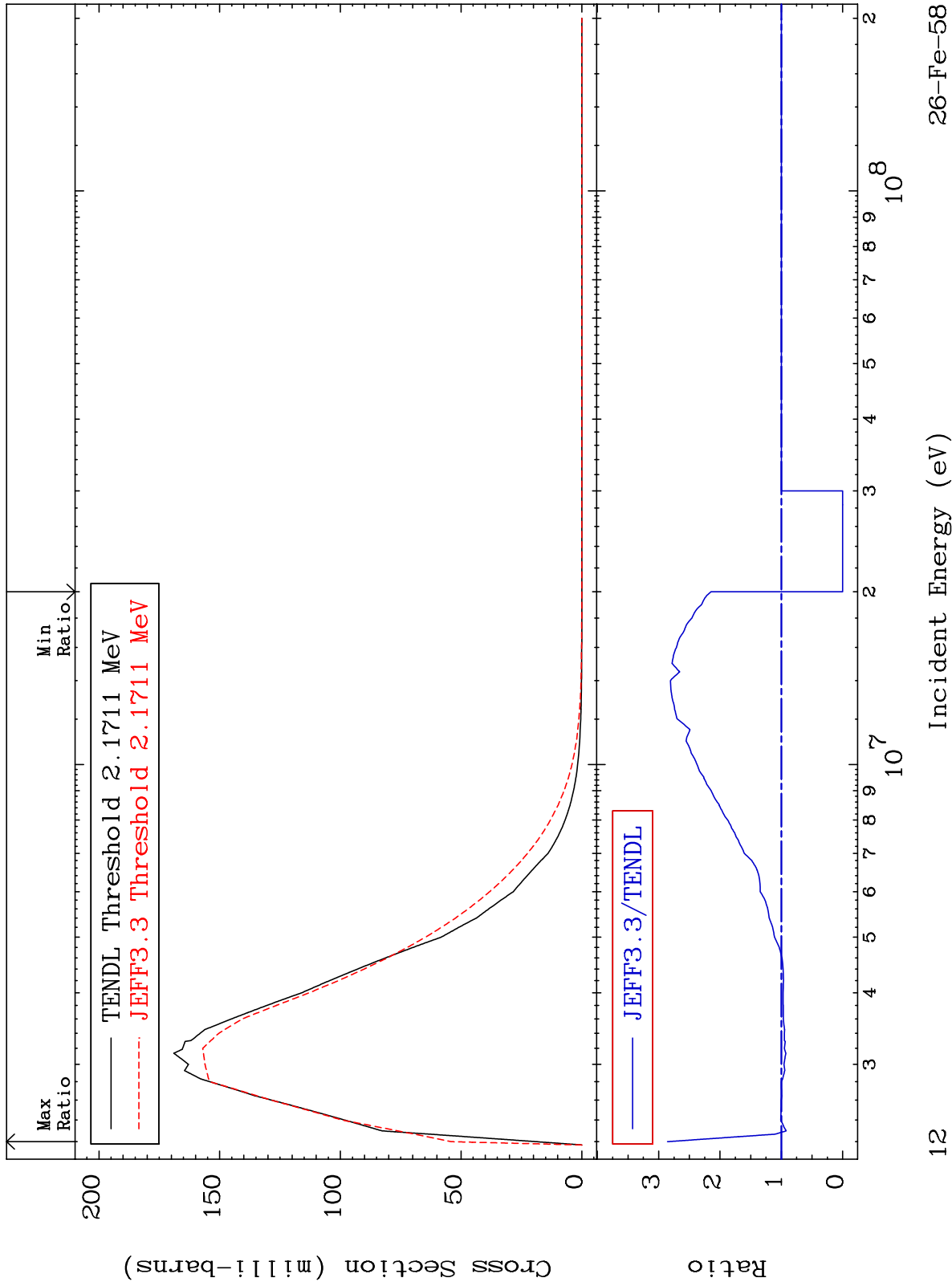
MAT 2637

MT= 53 (n,n') Level
Cross Section

26-Fe-58
-100.0 To 31.78 %



MAT 2637 MT= 54 (n,n') Level Cross Section -100.0 To 185.5 % 26-Fe-58



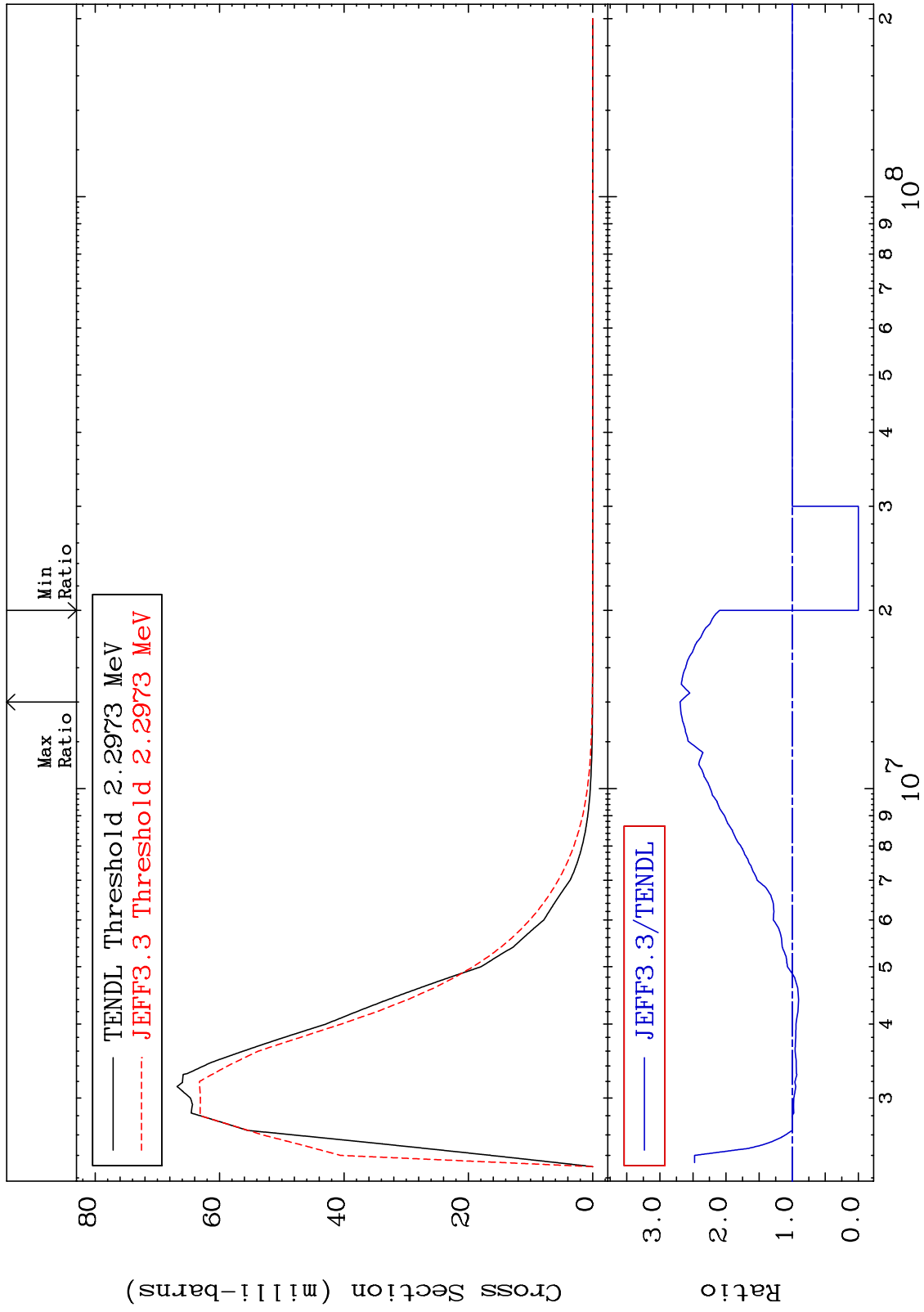
MAT 2637

MT= 55 (n, n') Level

²⁶Fe-58

-100.0 To 169.5 %

Cross Section



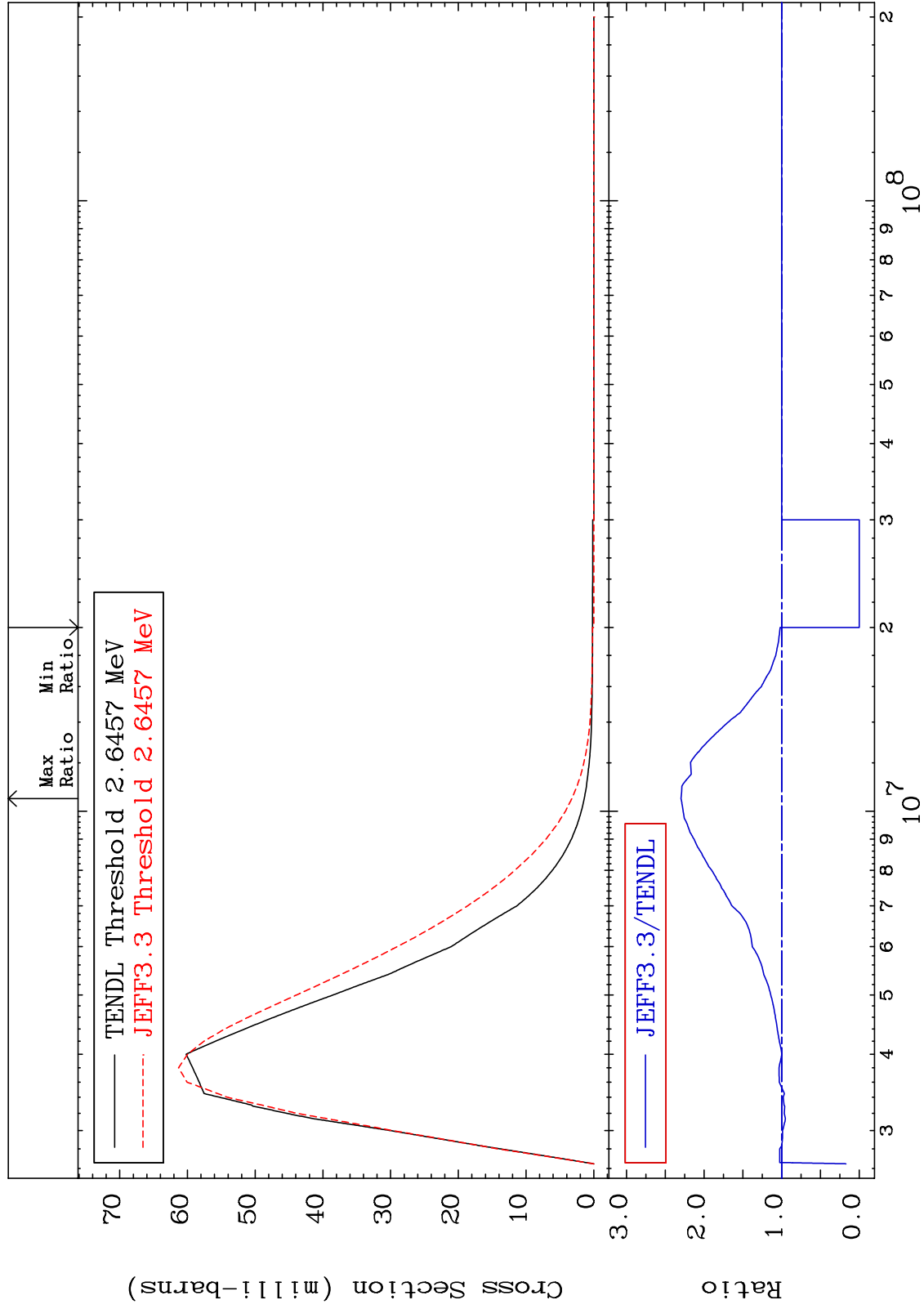
MAT 2637

MT= 56 (n,n') Level

26-Fe-58

-100.0 To 129.6 %

Cross Section



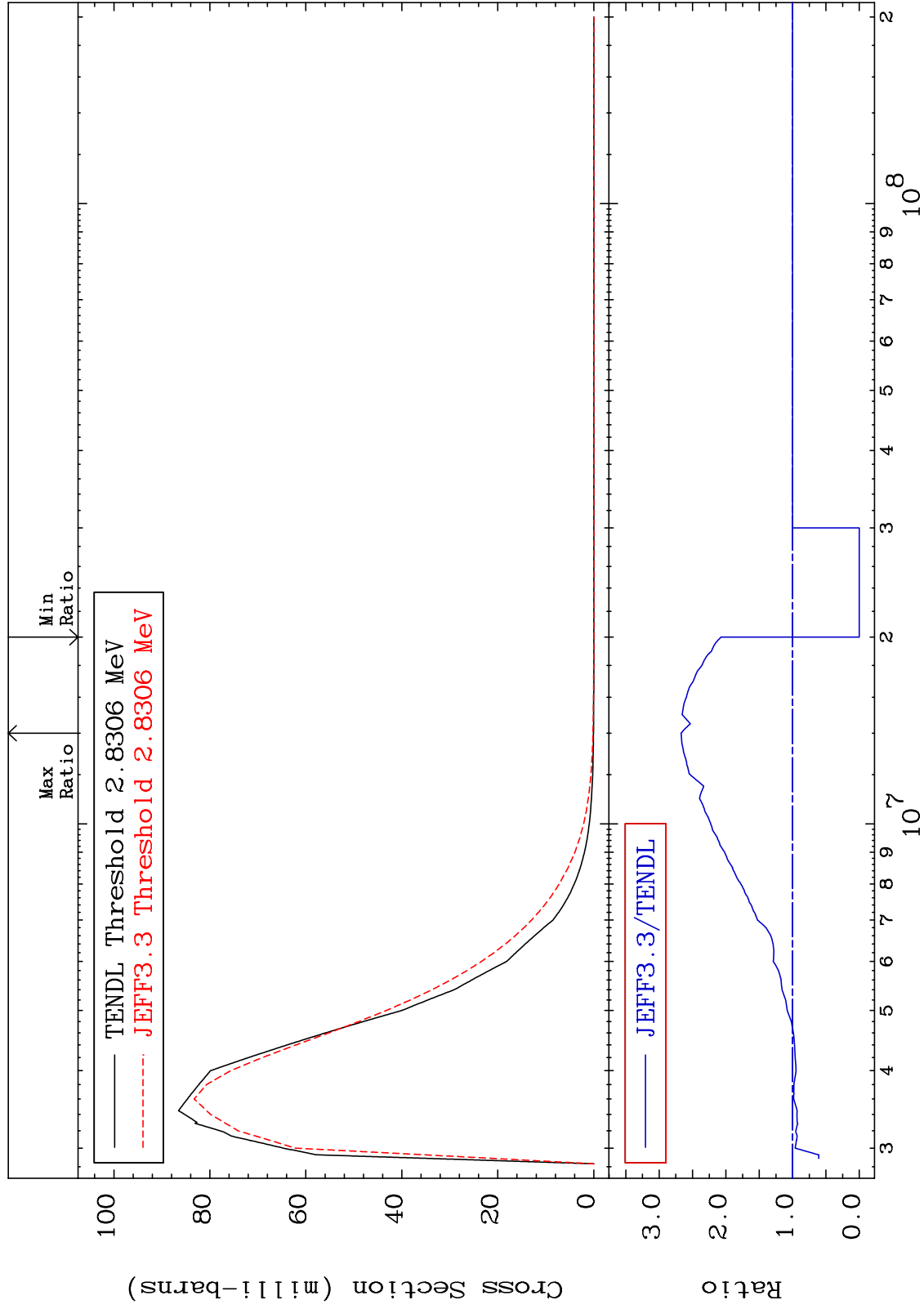
MAT 2637

MT= 57 (n, n') Level

²⁶Fe-58

-100.0 To 167.2 %

Cross Section



15

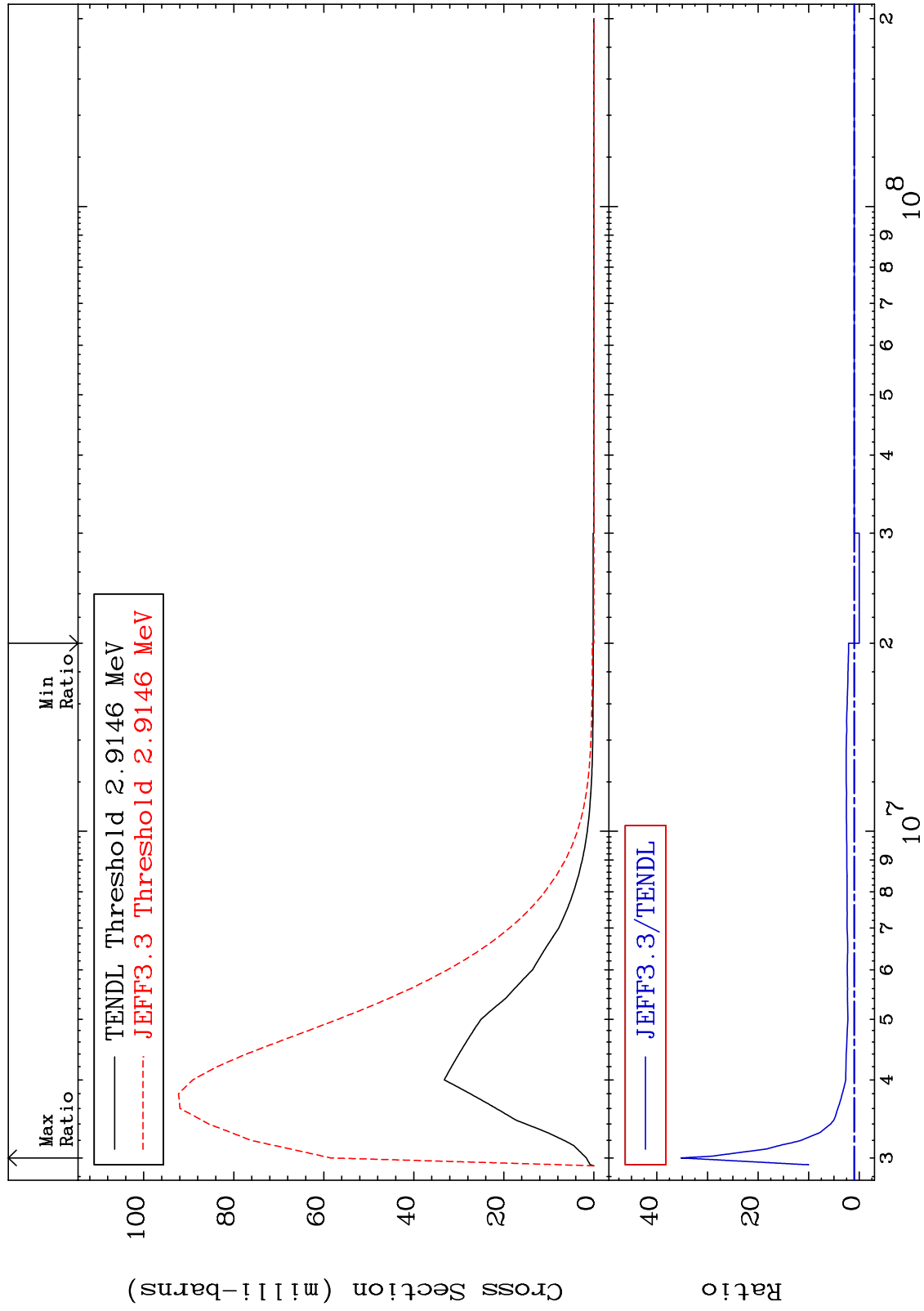
Incident Energy (eV)

²⁶Fe-58

MAT 2637

MT= 58 (n,n') Level
Cross Section

26-Fe-58
-100.0 To 3422. %



16

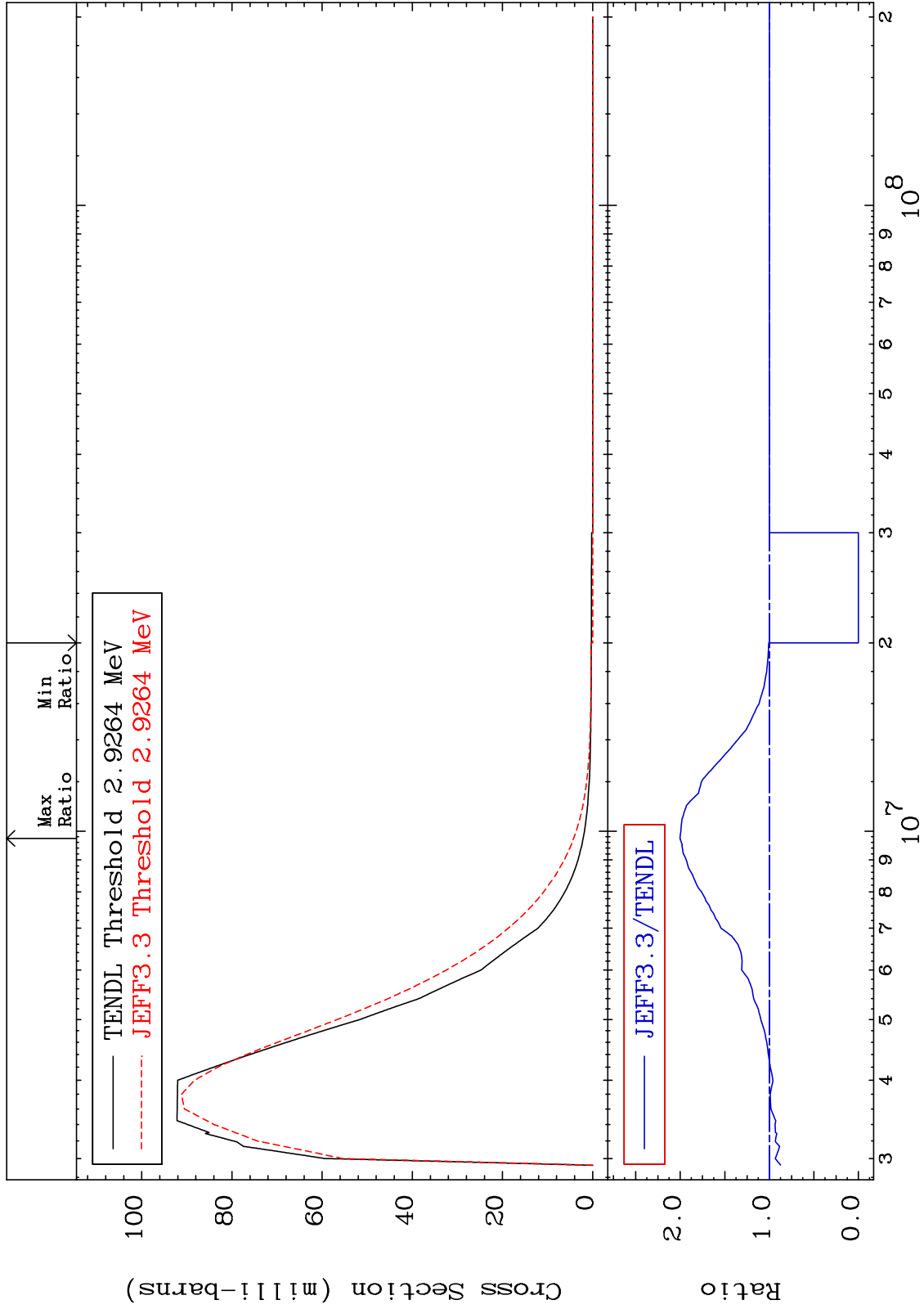
Incident Energy (eV)

26-Fe-58

MAT 2637

MT= 59 (n, n') Level
Cross Section

26-Fe-58
-100.0 To 100.3 %



17

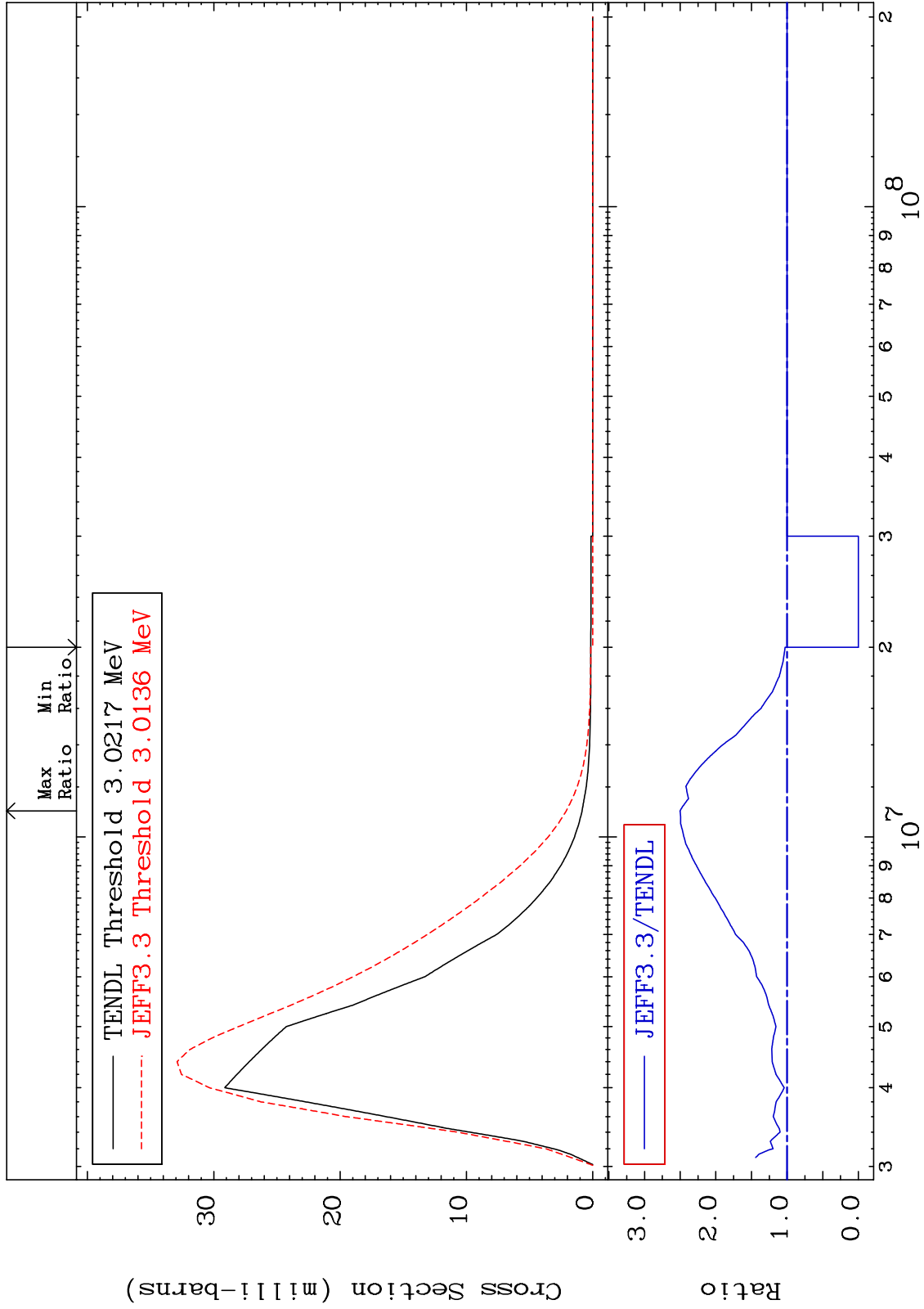
Incident Energy (eV)

26-Fe-58

MAT 2637

MT= 60 (n,n') Level
Cross Section

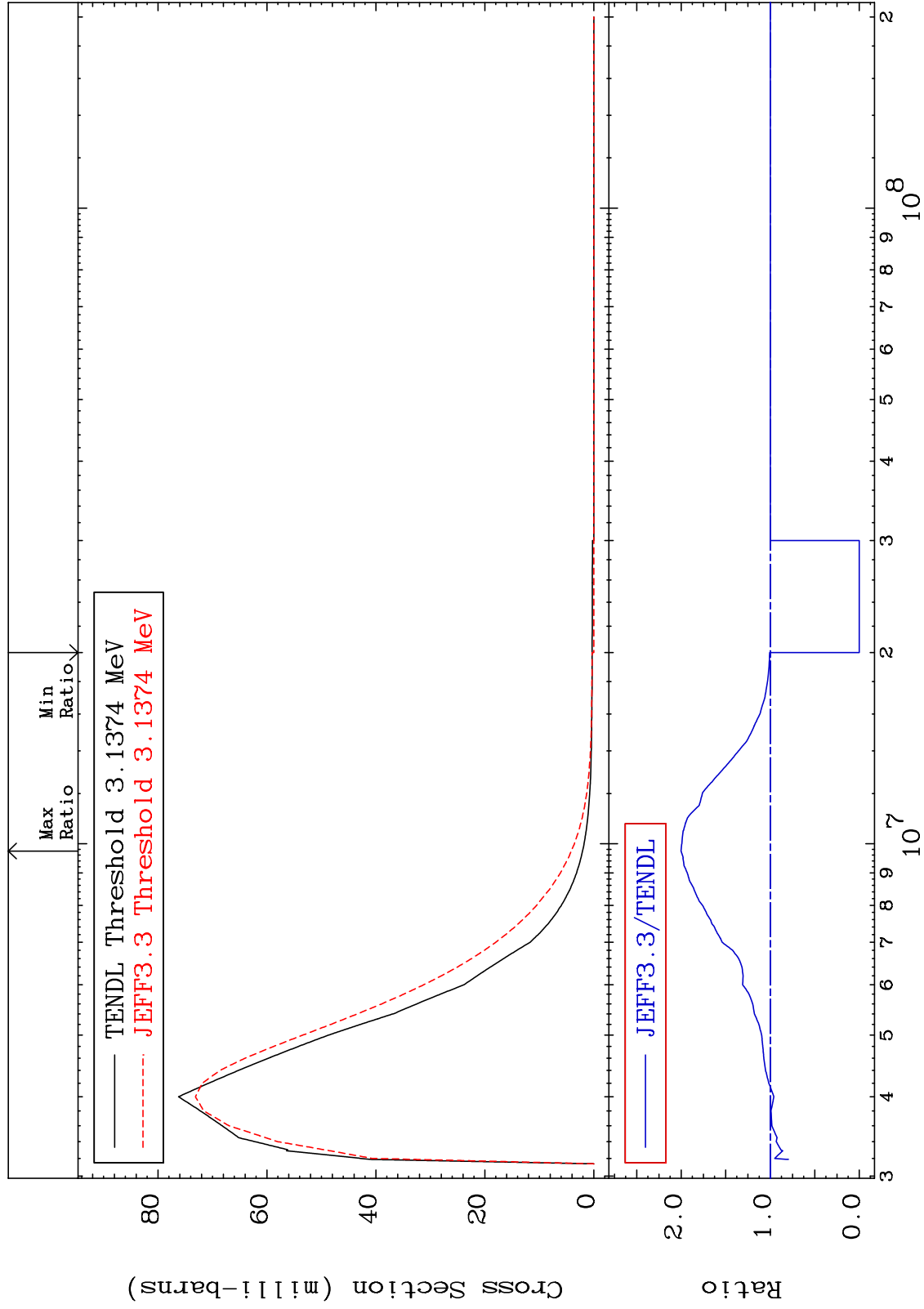
26-Fe-58
-100.0 To 150.2 %

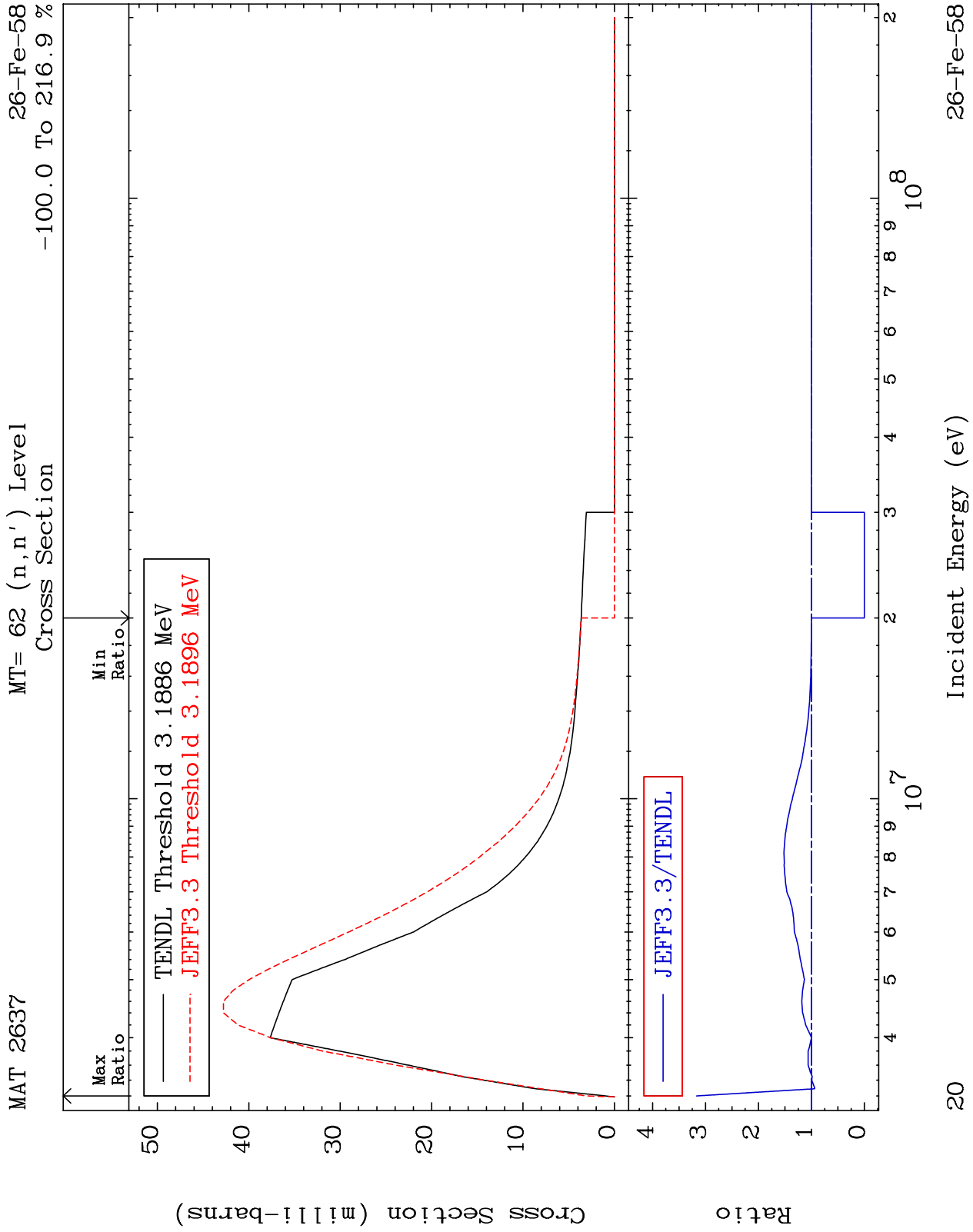


MAT 2637

MT= 61 (n,n') Level
Cross Section

26-Fe-58
-100.0 To 100.2 %

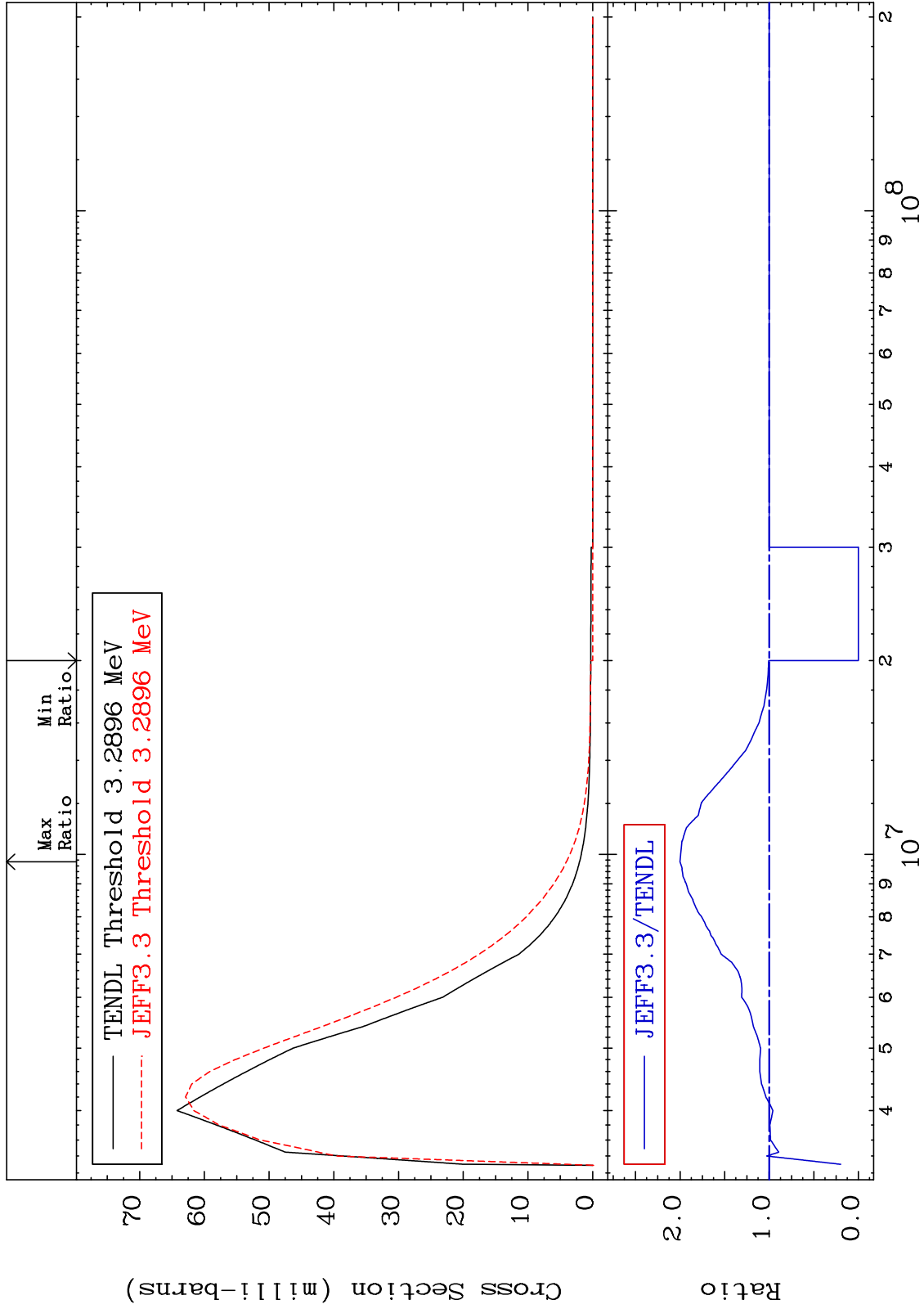




MAT 2637

MT= 63 (n,n') Level
Cross Section

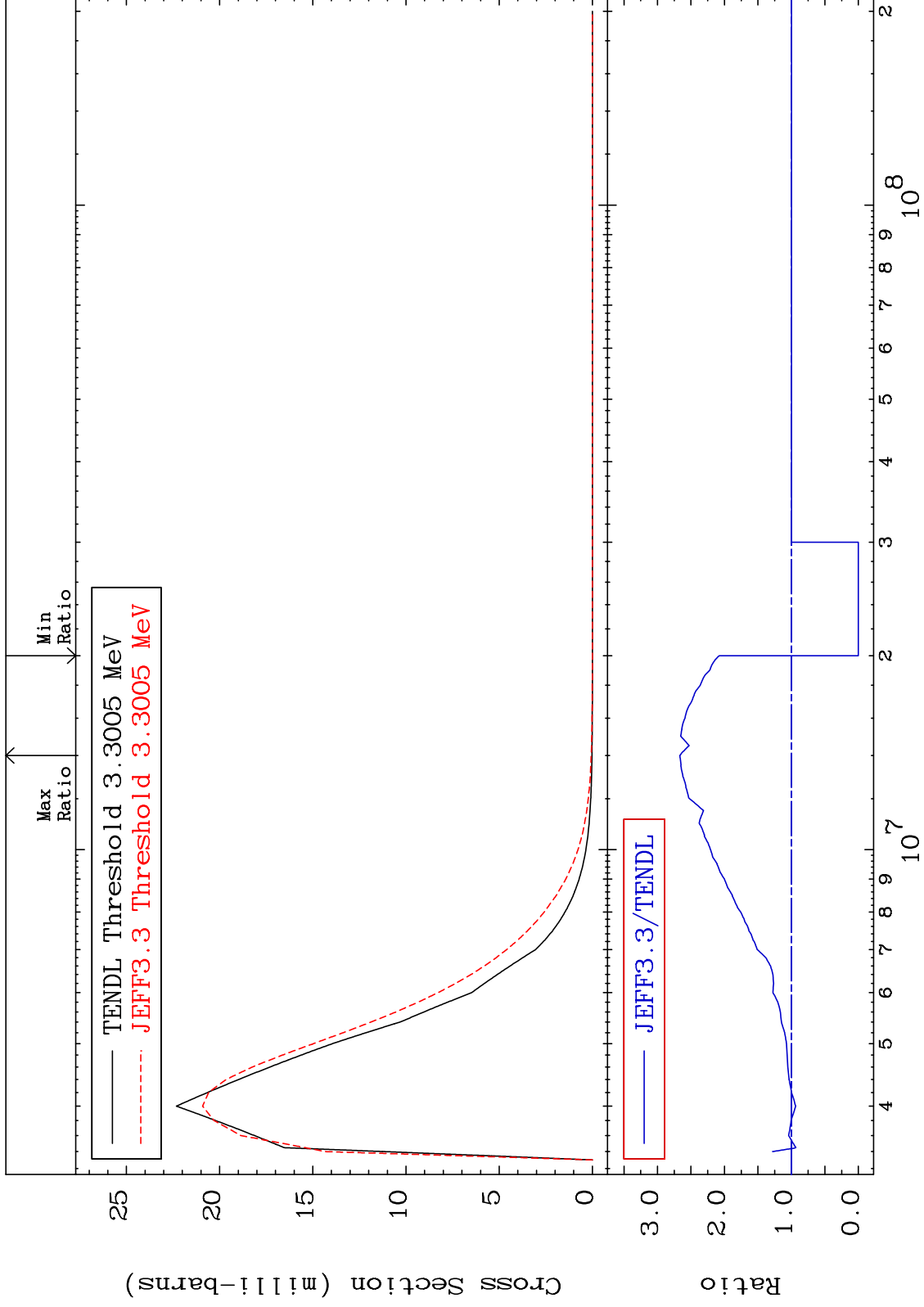
26-Fe-58
-100.0 To 100.1 %



MAT 2637

MT= 64 (n,n') Level
Cross Section

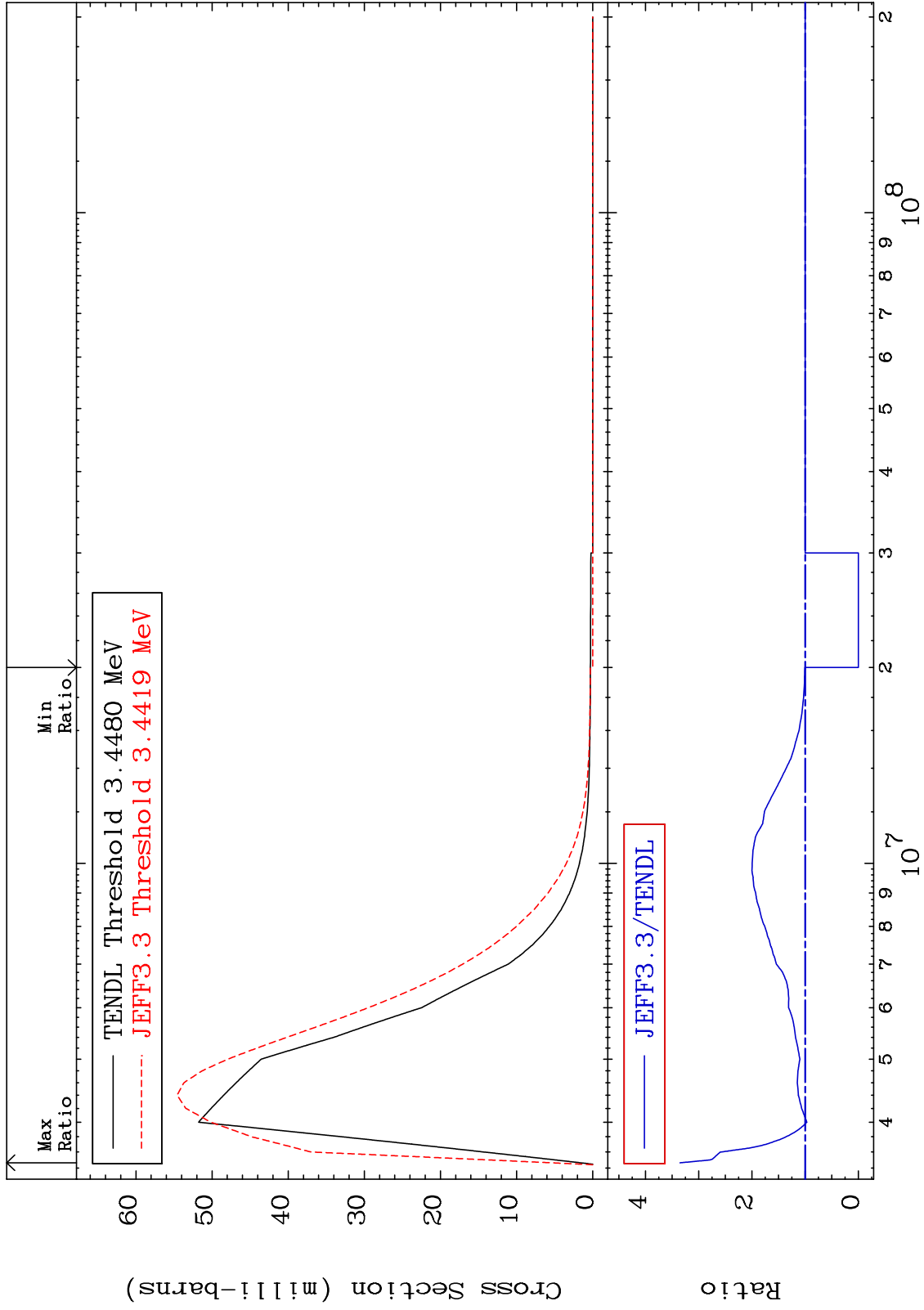
26-Fe-58
-100.0 To 166.7 %



MAT 2637

MT= 65 (n,n') Level
Cross Section

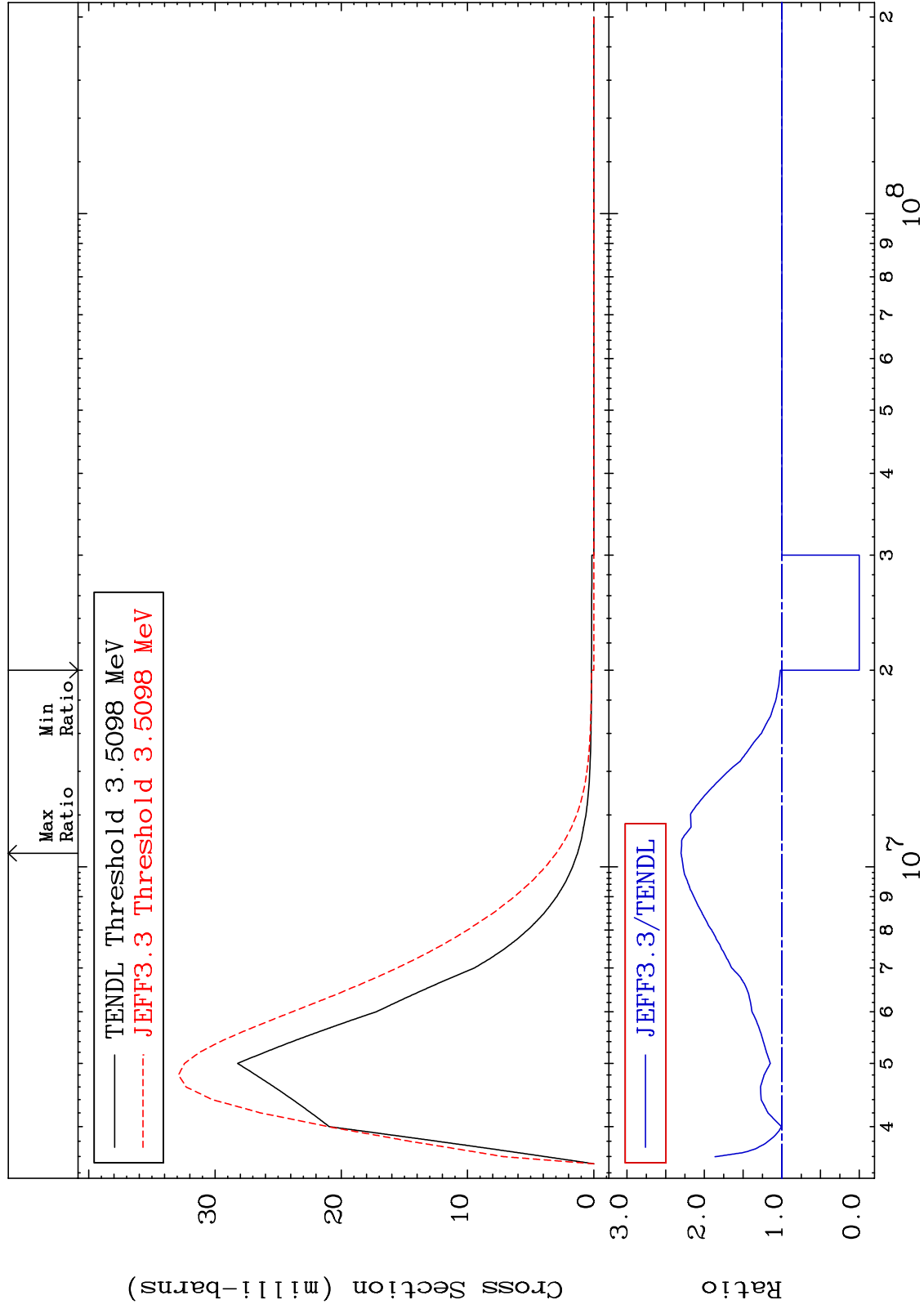
26-Fe-58
-100.0 To 235.1 %



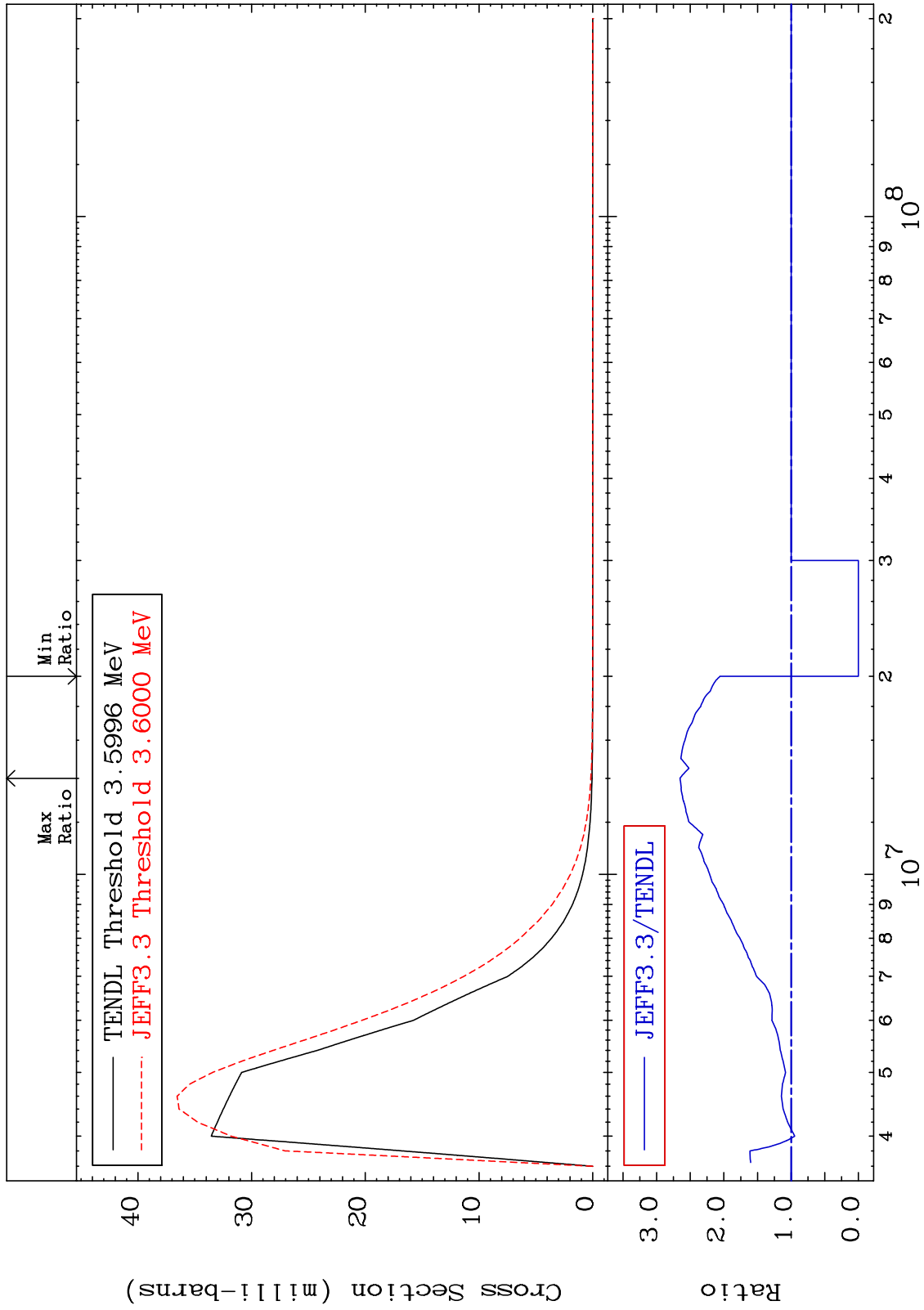
MAT 2637

MT= 66 (n,n') Level
Cross Section

26-Fe-58
-100.0 To 130.1 %



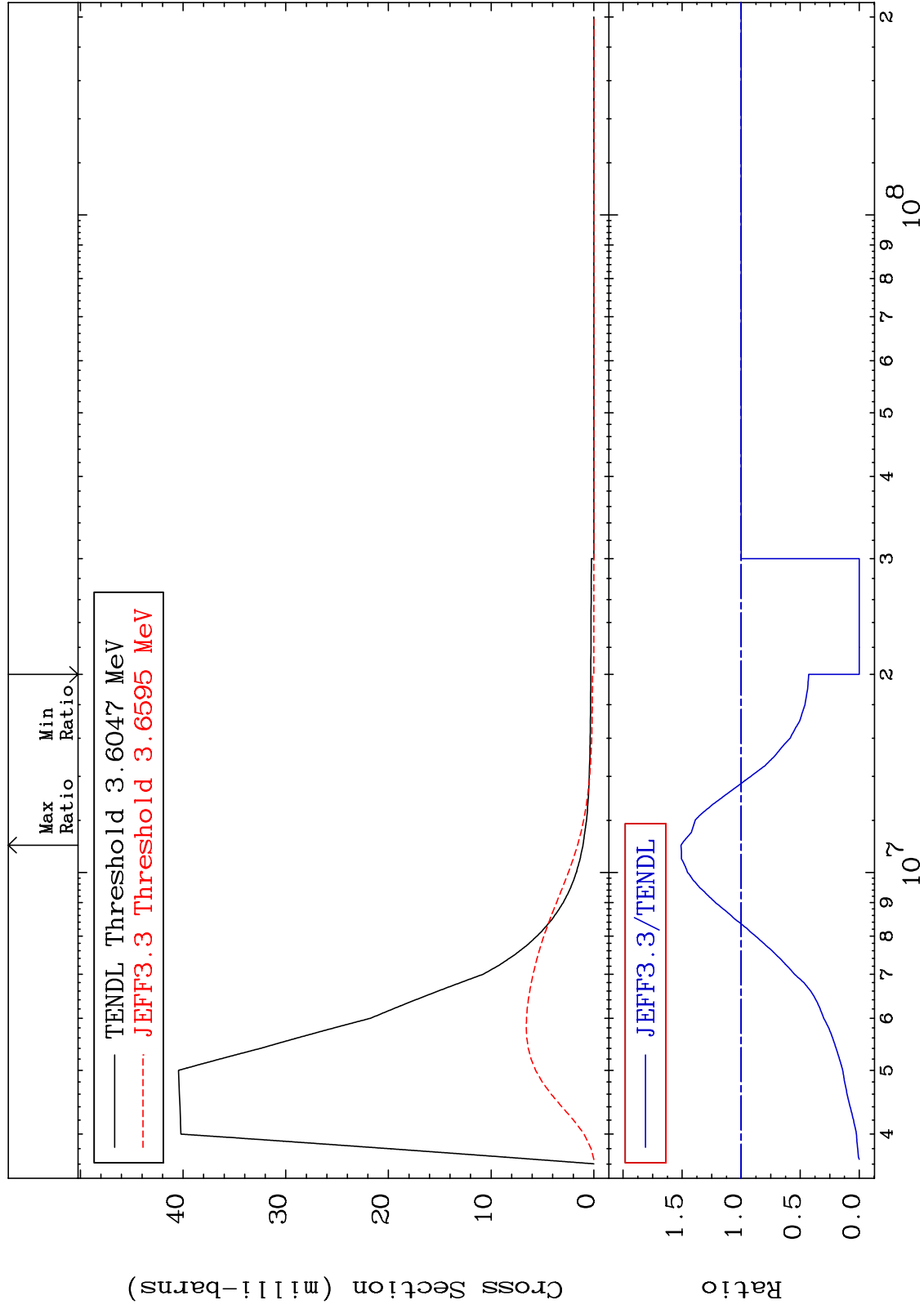
MAT 2637 MT= 67 (n,n') Level Cross Section -100.0 To 165.2 % 26-Fe-58



MAT 2637

MT= 68 (n,n') Level
Cross Section

26-Fe-58
-100.0 To 50.93 %



26

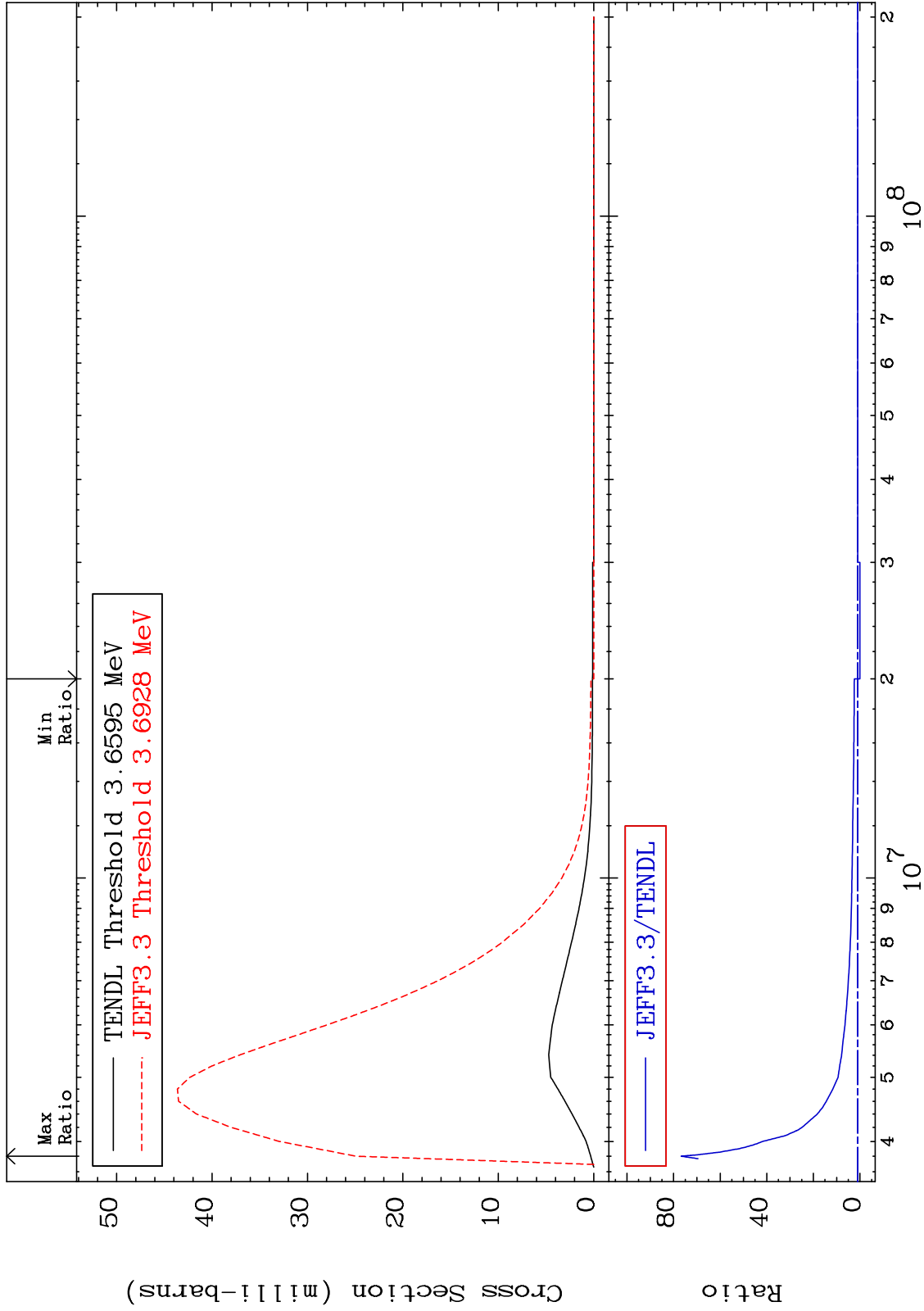
Incident Energy (eV)

26-Fe-58

MAT 2637

MT= 69 (n,n') Level
Cross Section

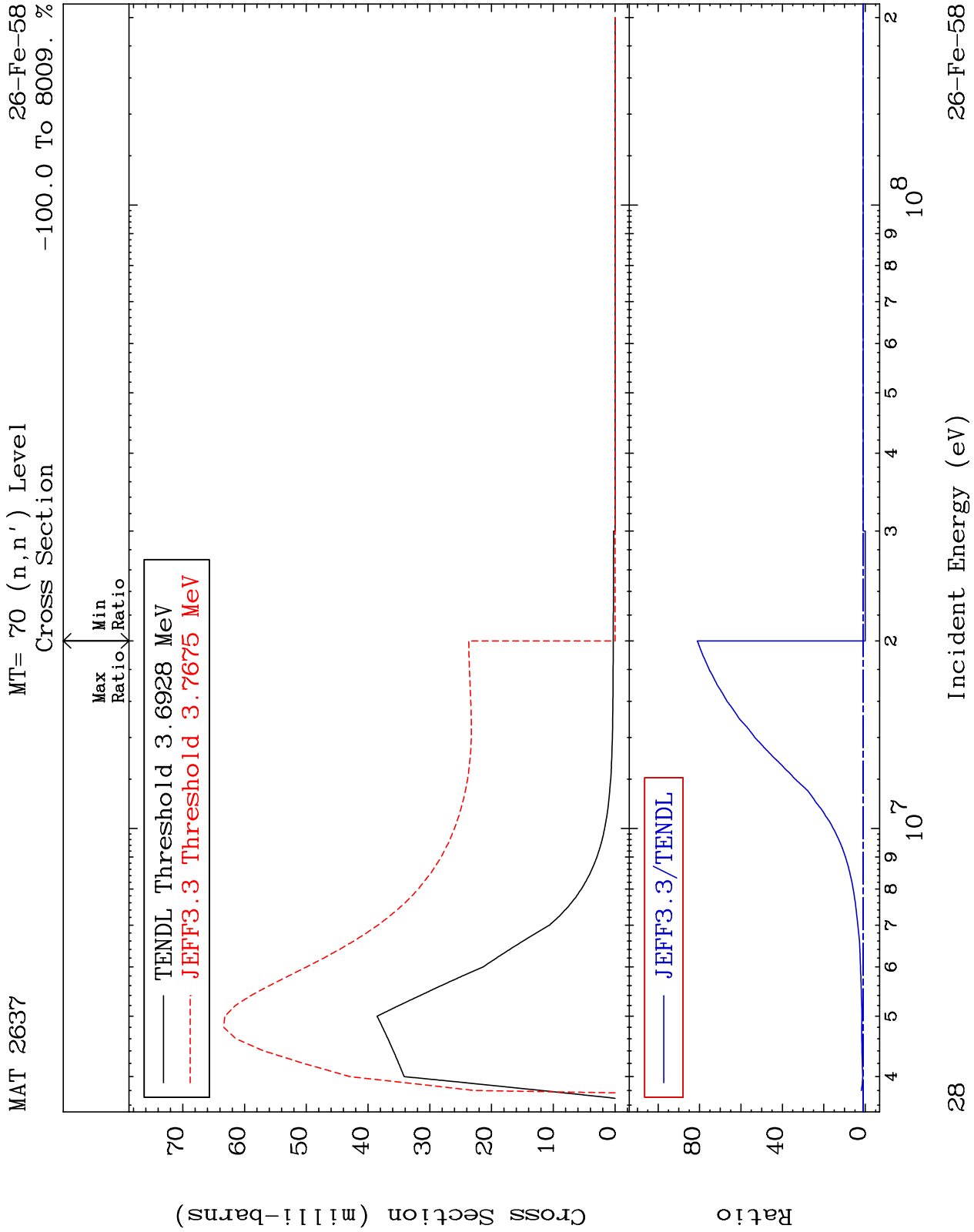
26-Fe-58
-100.0 To 7572. %



27

Incident Energy (eV)

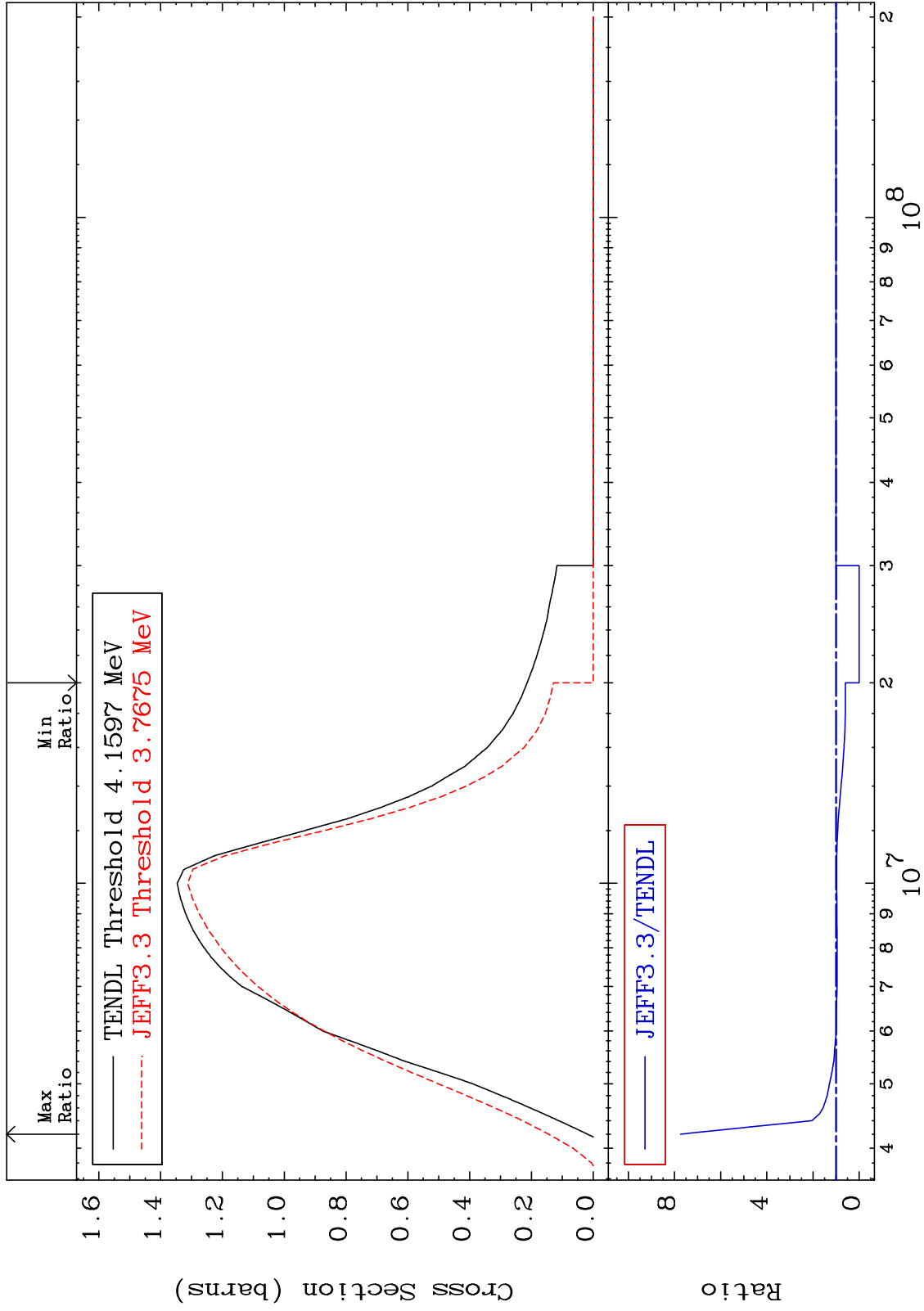
26-Fe-58



MAT 2637

(n, n') Continuum
Cross Section

²⁶Fe-58
-100.0 To 673.6 %



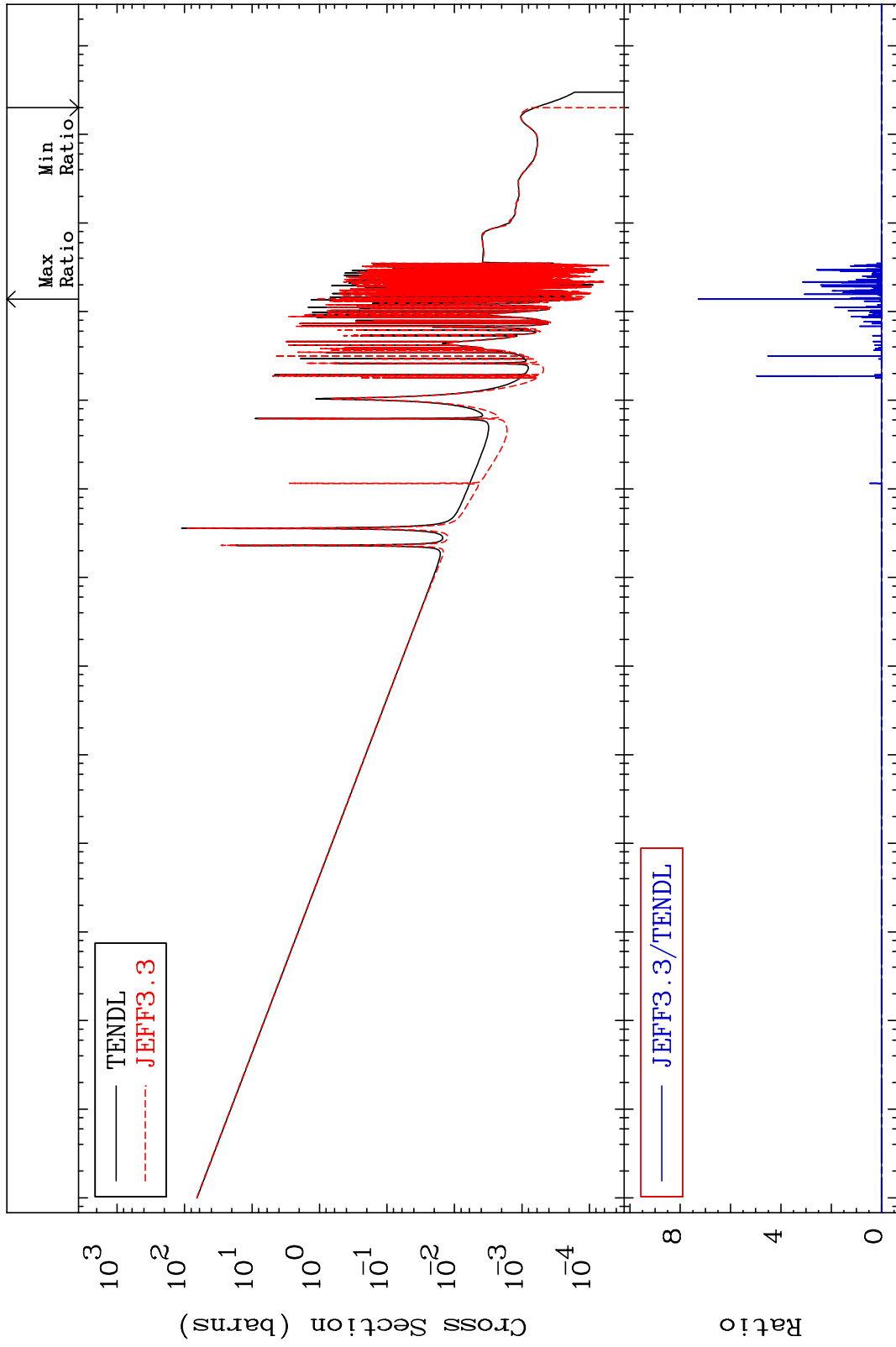
MAT 2637

(n, γ)

²⁶Fe-58

Cross Section

-100.0 To 9999. %



Incident Energy (eV)

²⁶Fe-58

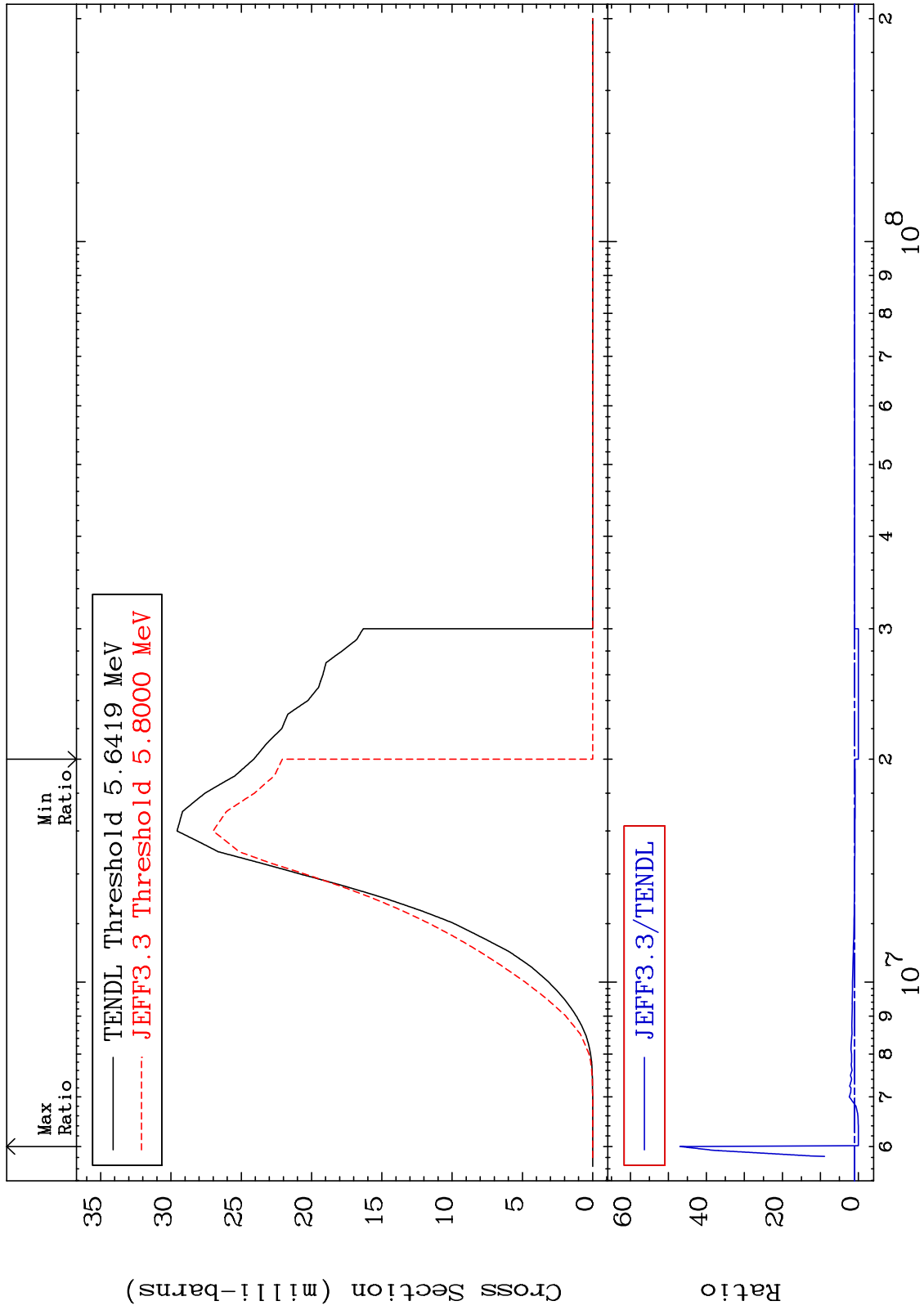
MAT 2637

(n,p)

²⁶Fe-58

Cross Section

-100.0 To 4594. %



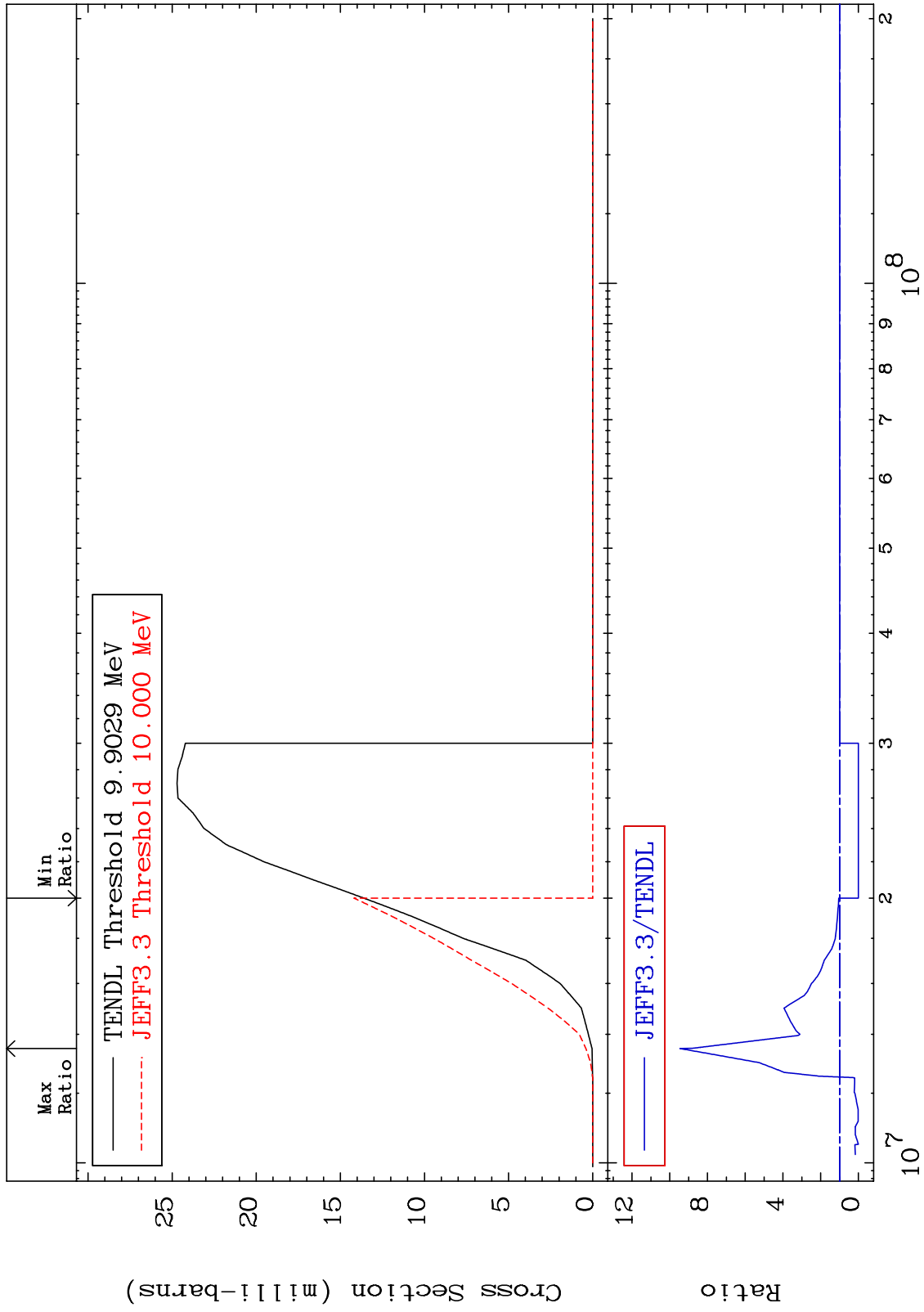
MAT 2637

(n, d)

²⁶Fe-58

Cross Section

-100.0 To 845.4 %



32

32

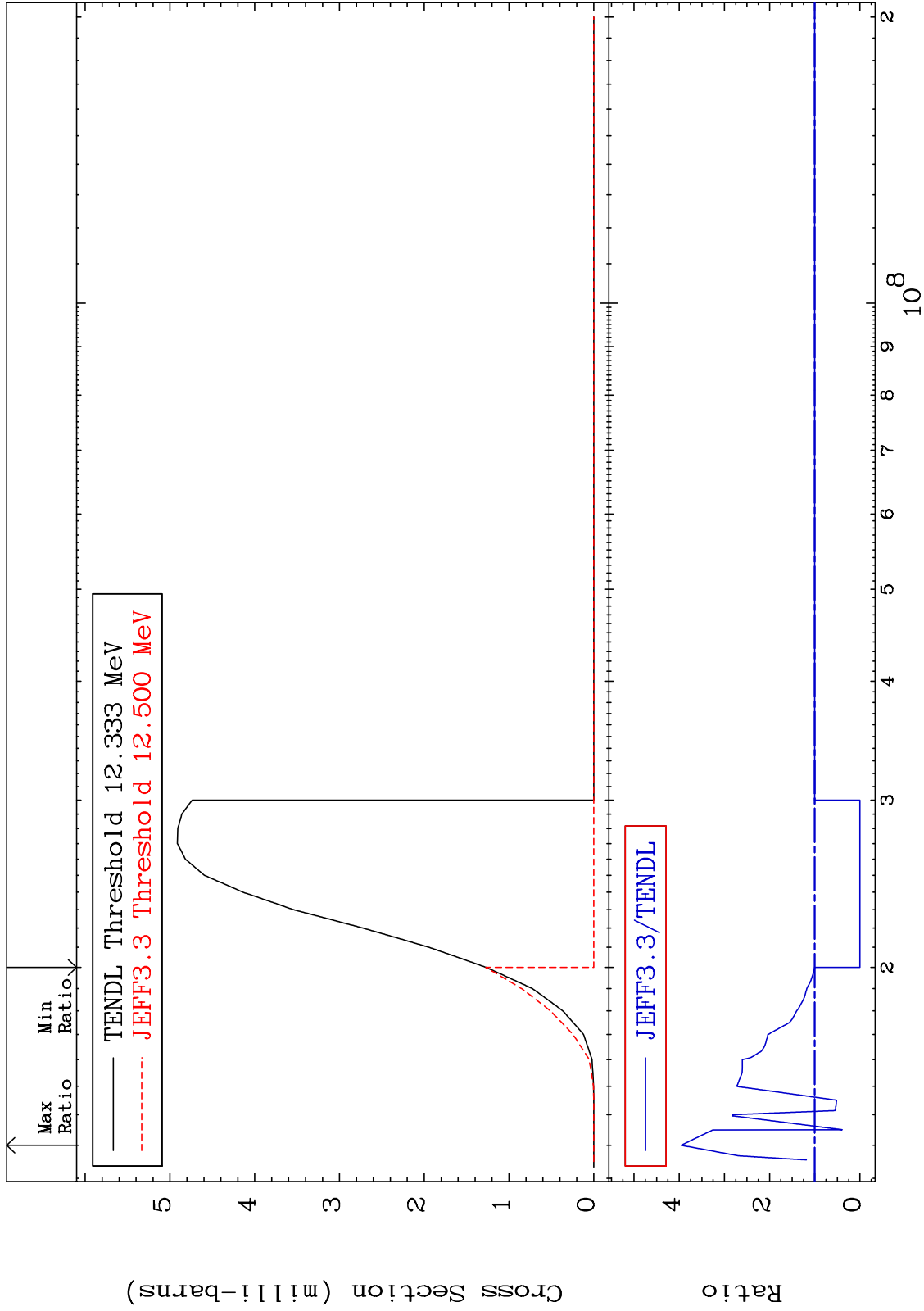
MAT 2637

(n, t)

²⁶Fe-58

Cross Section

-100.0 To 295.6 %



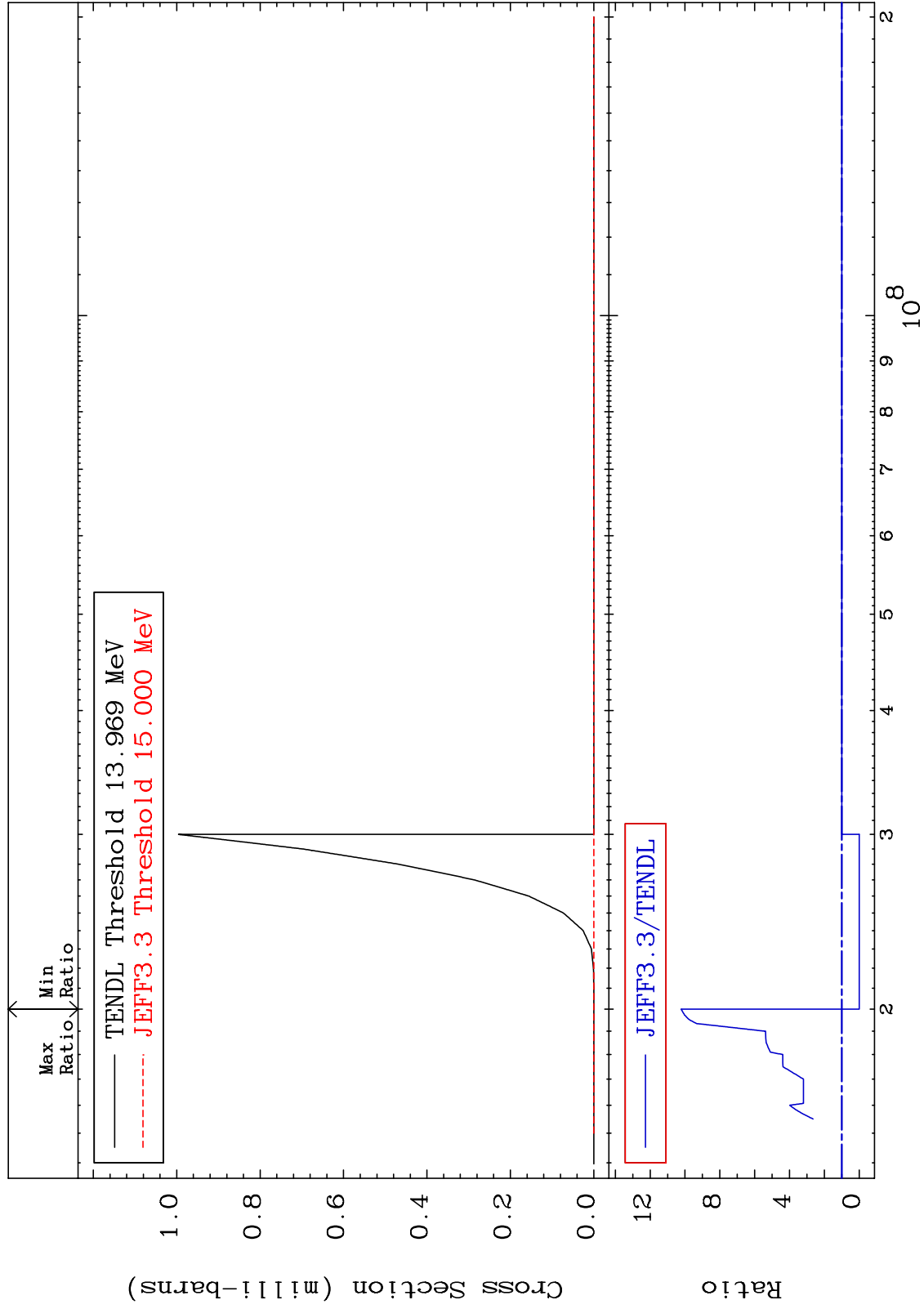
MAT 2637

(n, He-3)

²⁶Fe-58

Cross Section

-100.0 To 922.7 %



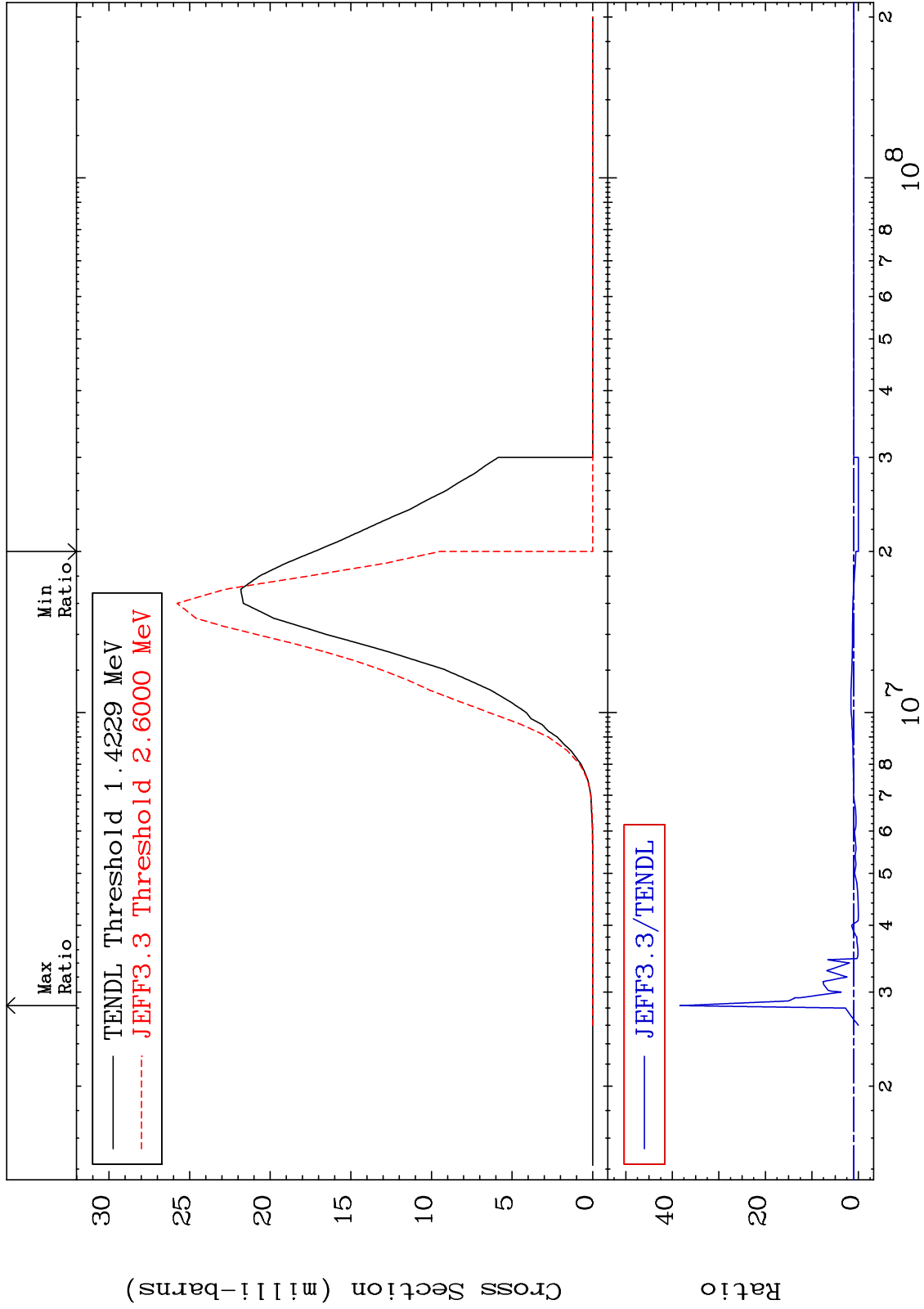
MAT 2637

(n, α)

²⁶Fe-58

Cross Section

-100.0 To 3734. %



35

Incident Energy (eV)

²⁶Fe-58

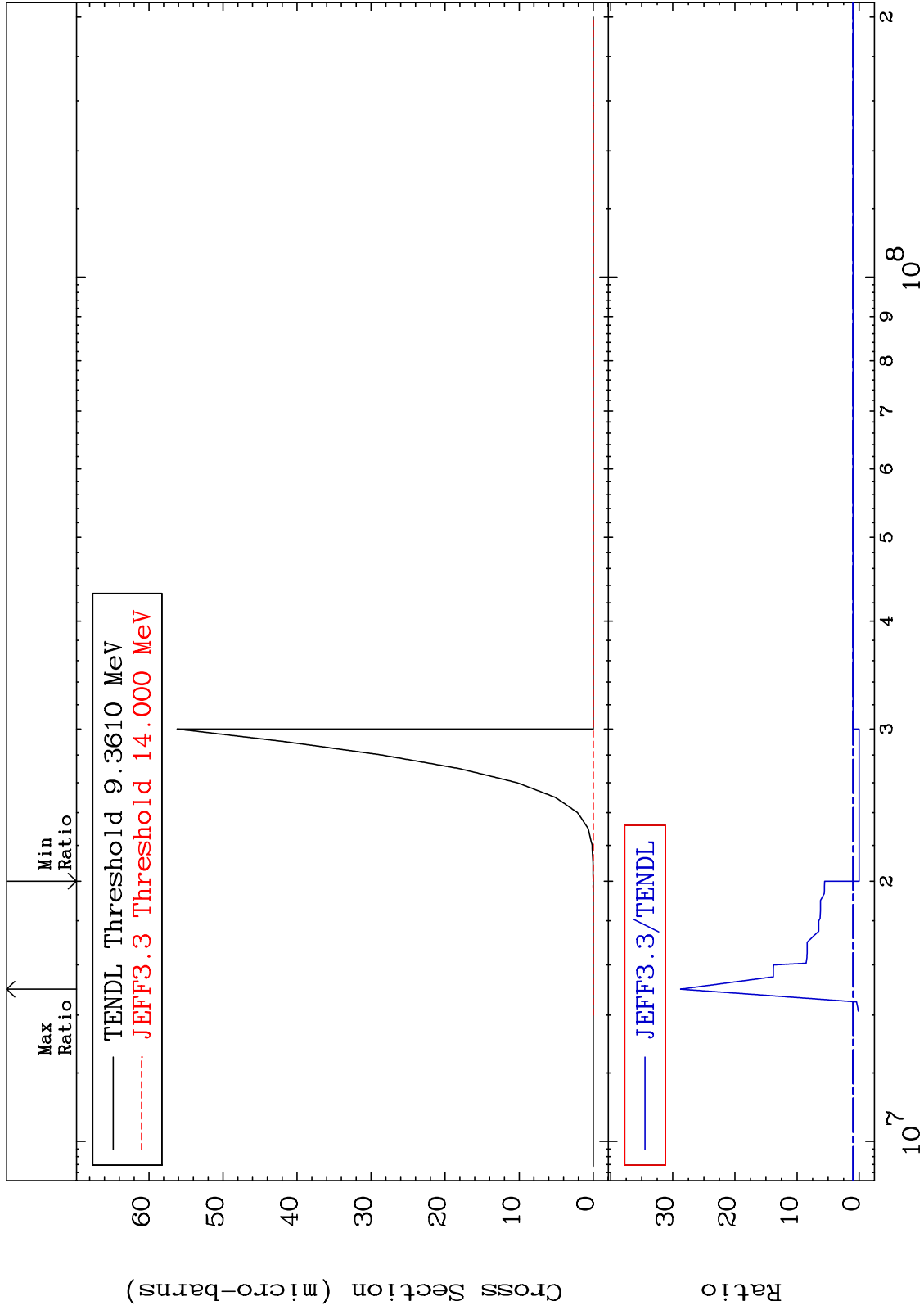
MAT 2637

(n, 2α)

²⁶Fe-58

Cross Section

-100.0 To 2774. %



36

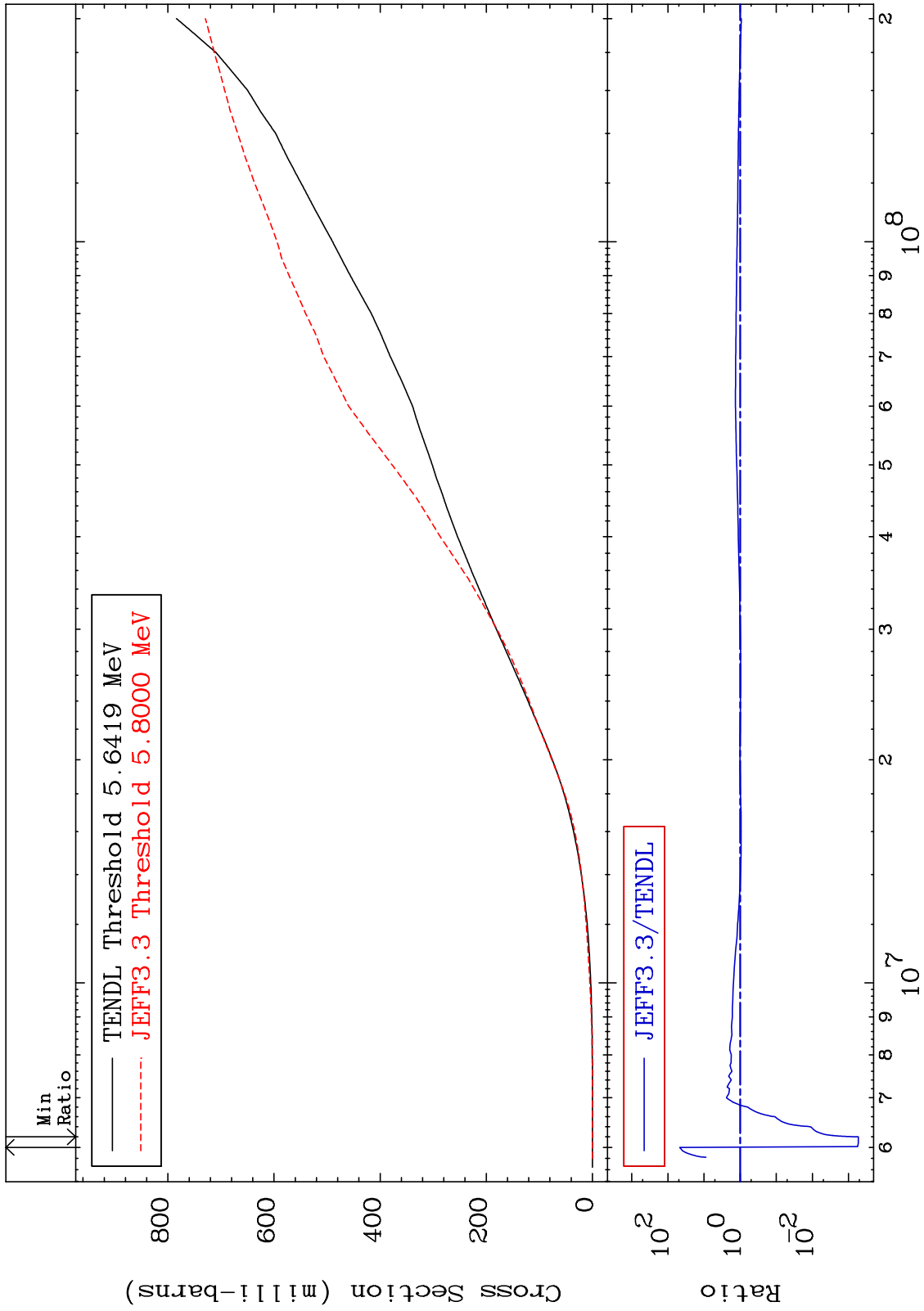
Incident Energy (eV)

²⁶Fe-58

MAT 2637

Hydrogen Production
Cross Section

26-Fe-58
-99.95 To 4594. %



37

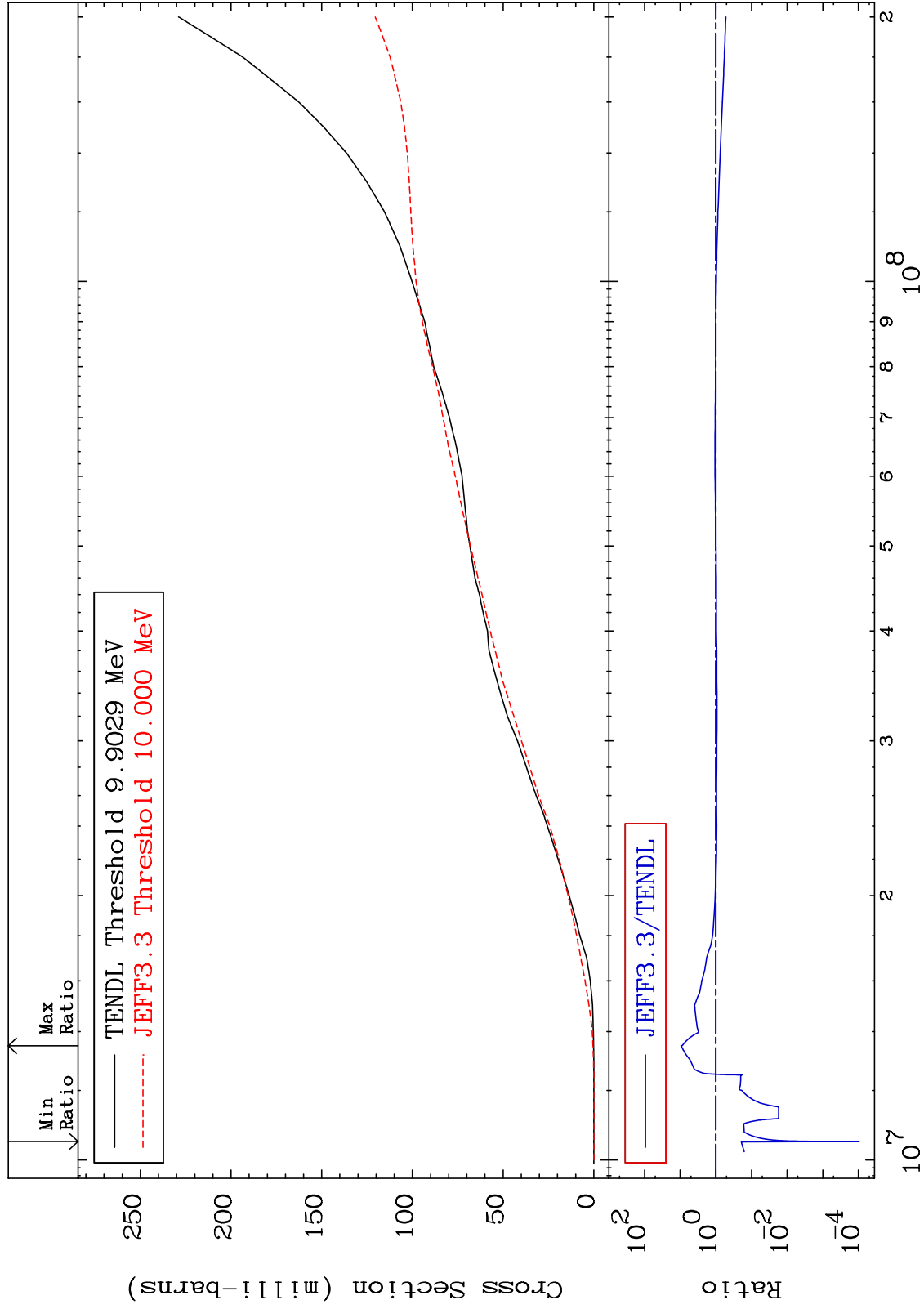
Incident Energy (eV)

26-Fe-58

MAT 2637

Deuterium Production
Cross Section

$^{26}\text{Fe-58}$
-99.99 To 845.4 %



Incident Energy (eV)

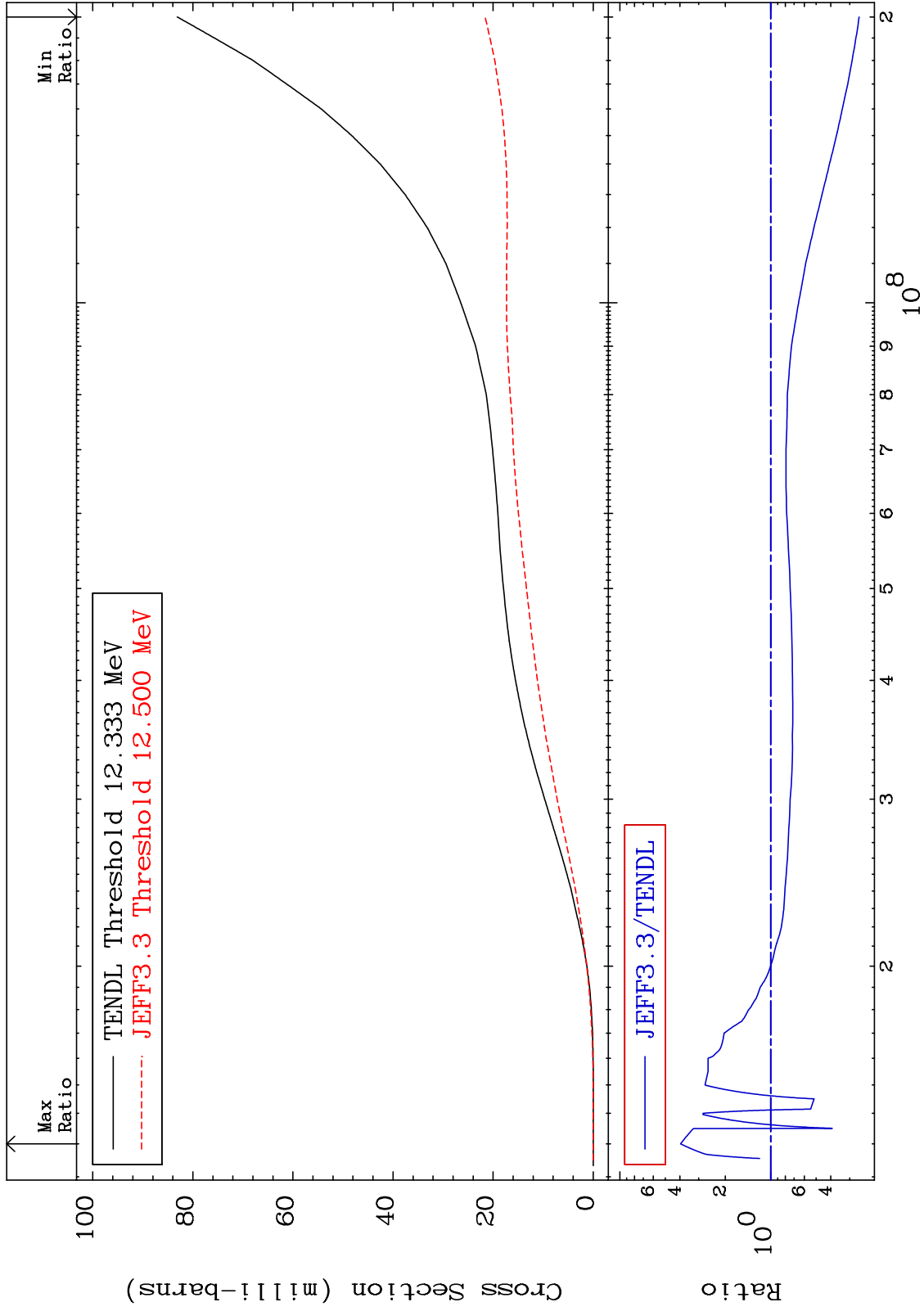
$^{26}\text{Fe-58}$

38

MAT 2637

Tritium Production
Cross Section

²⁶Fe-58
-74.04 To 295.6 %



39

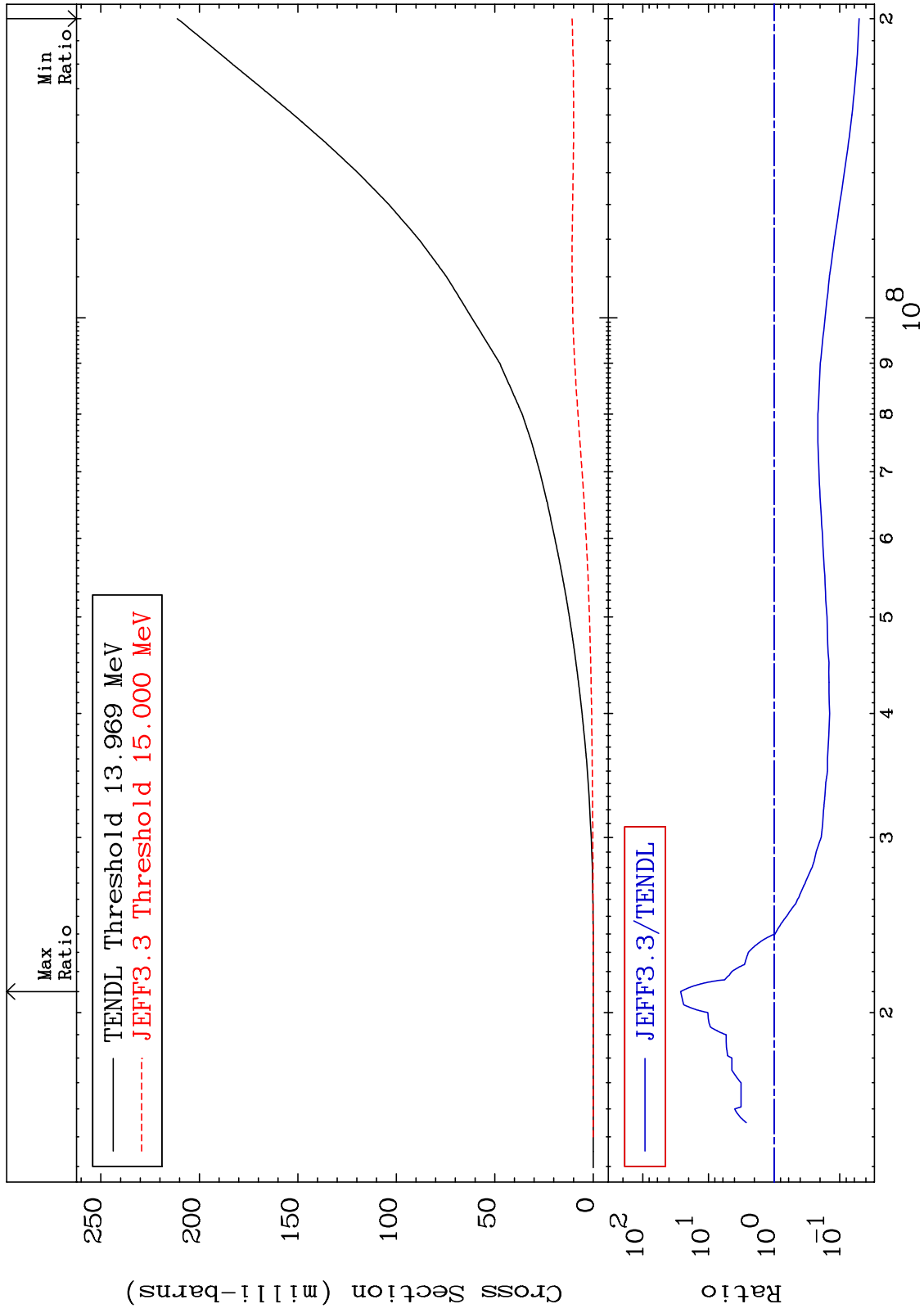
Incident Energy (eV)

²⁶Fe-58

MAT 2637

He-3 Production
Cross Section

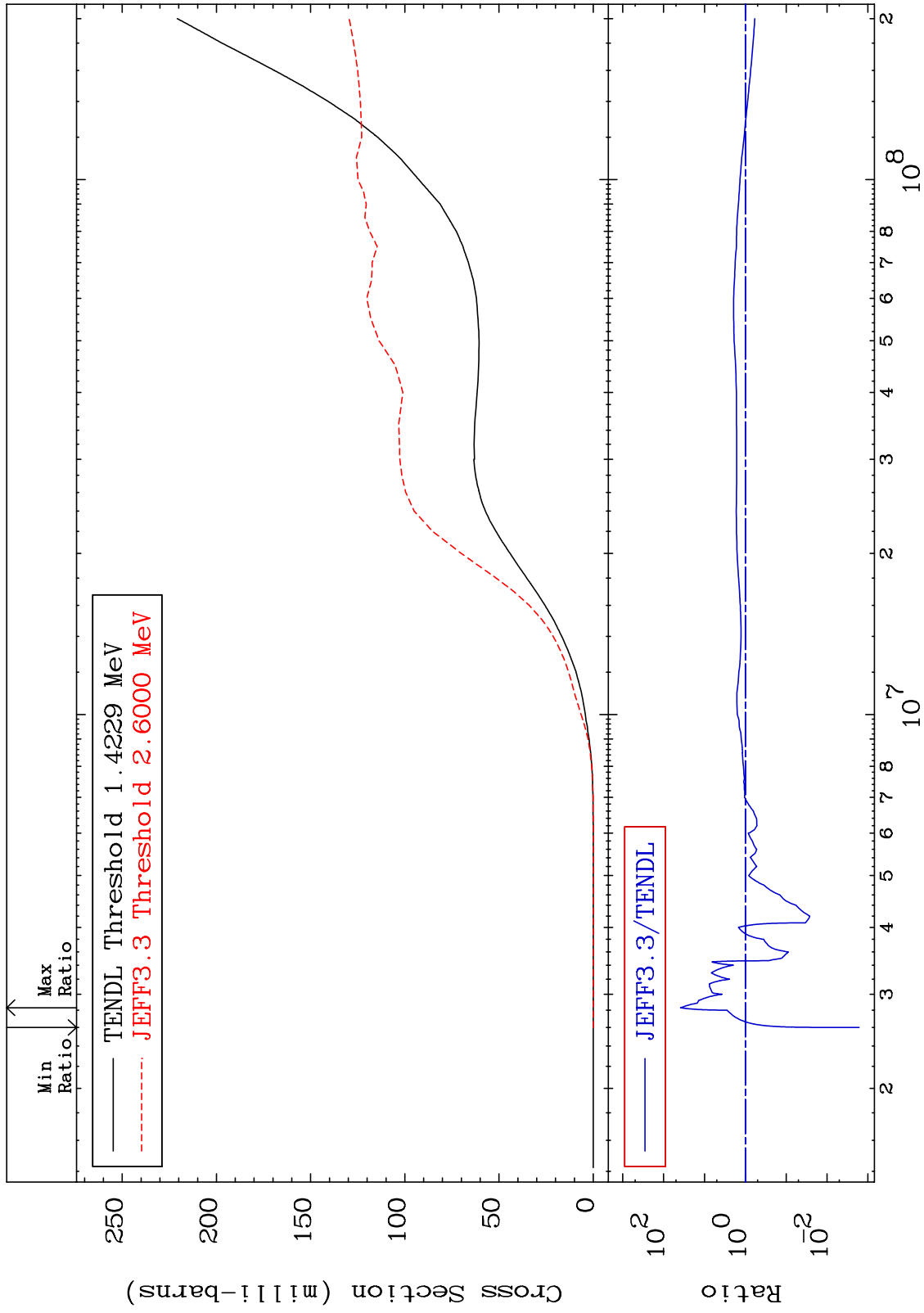
26-Fe-58
-94.93 To 2552. %



MAT 2637

He-4 Production
Cross Section

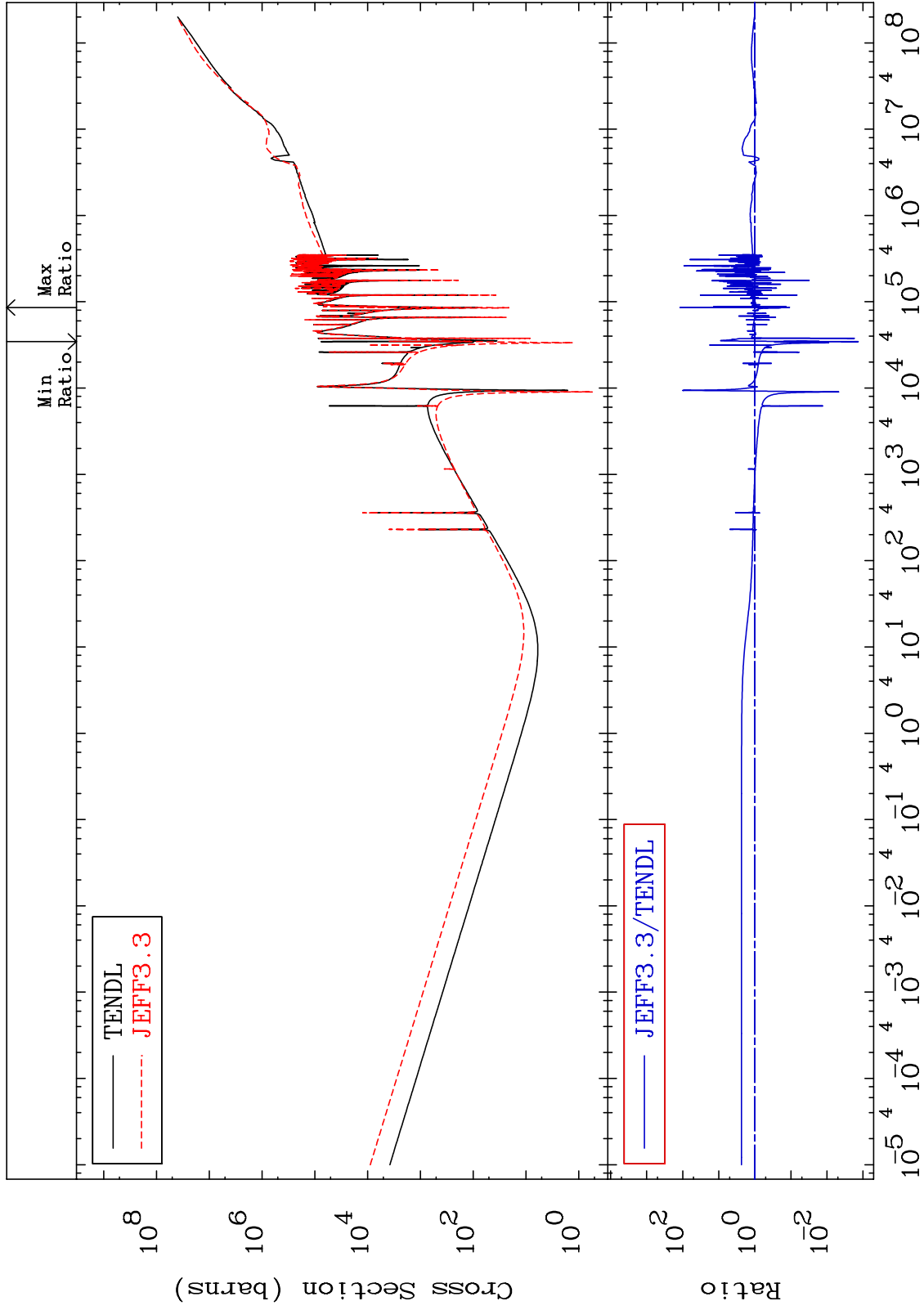
²⁶Fe-58
-99.84 To 3734. %



MAT 2637

Kerma total (eV-barns)
Cross Section

26-Fe-58
-99.87 To 9999. %



42

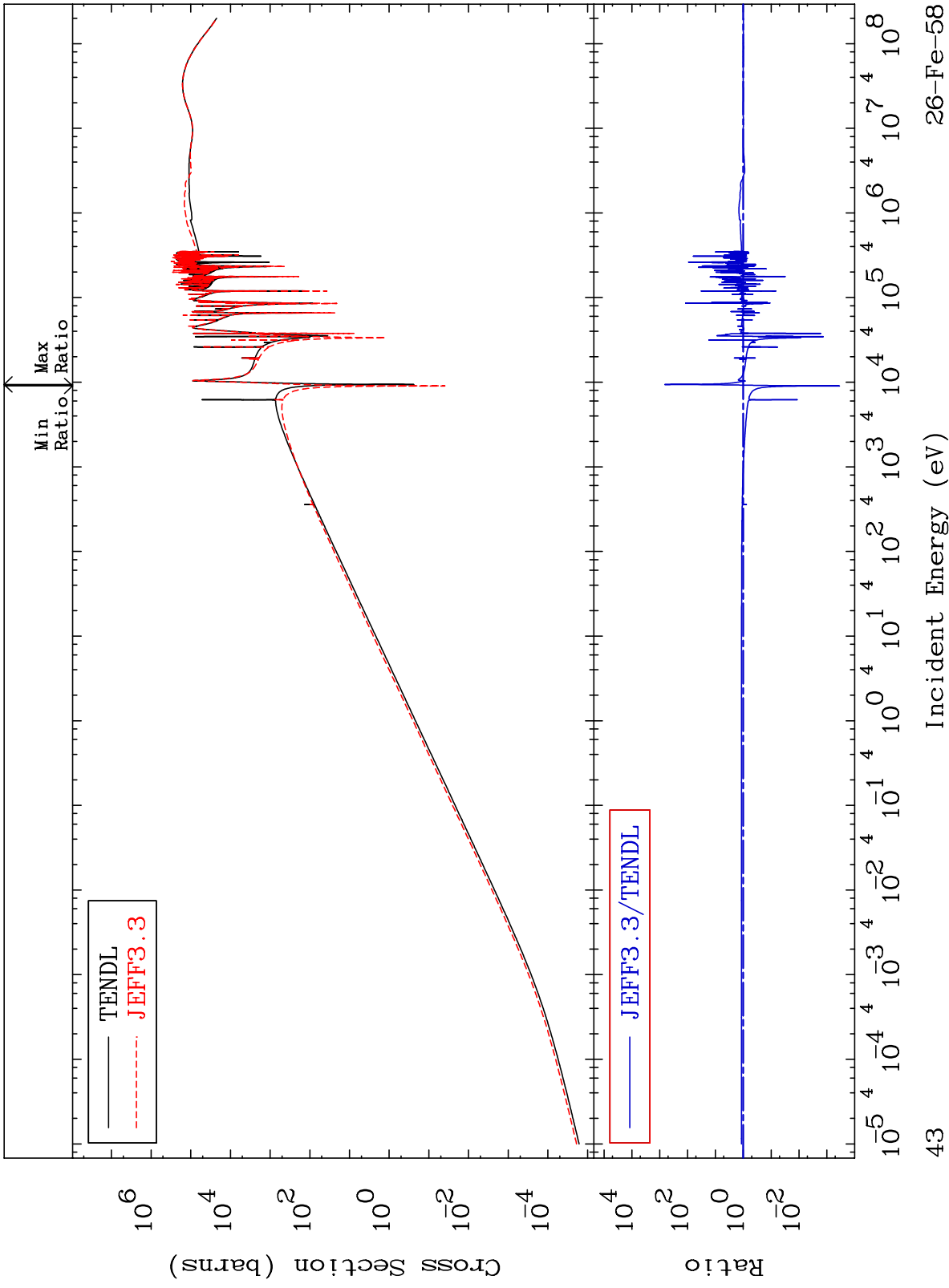
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma elastic
Cross Section

26-Fe-58
-99.97 To 9999. %



43

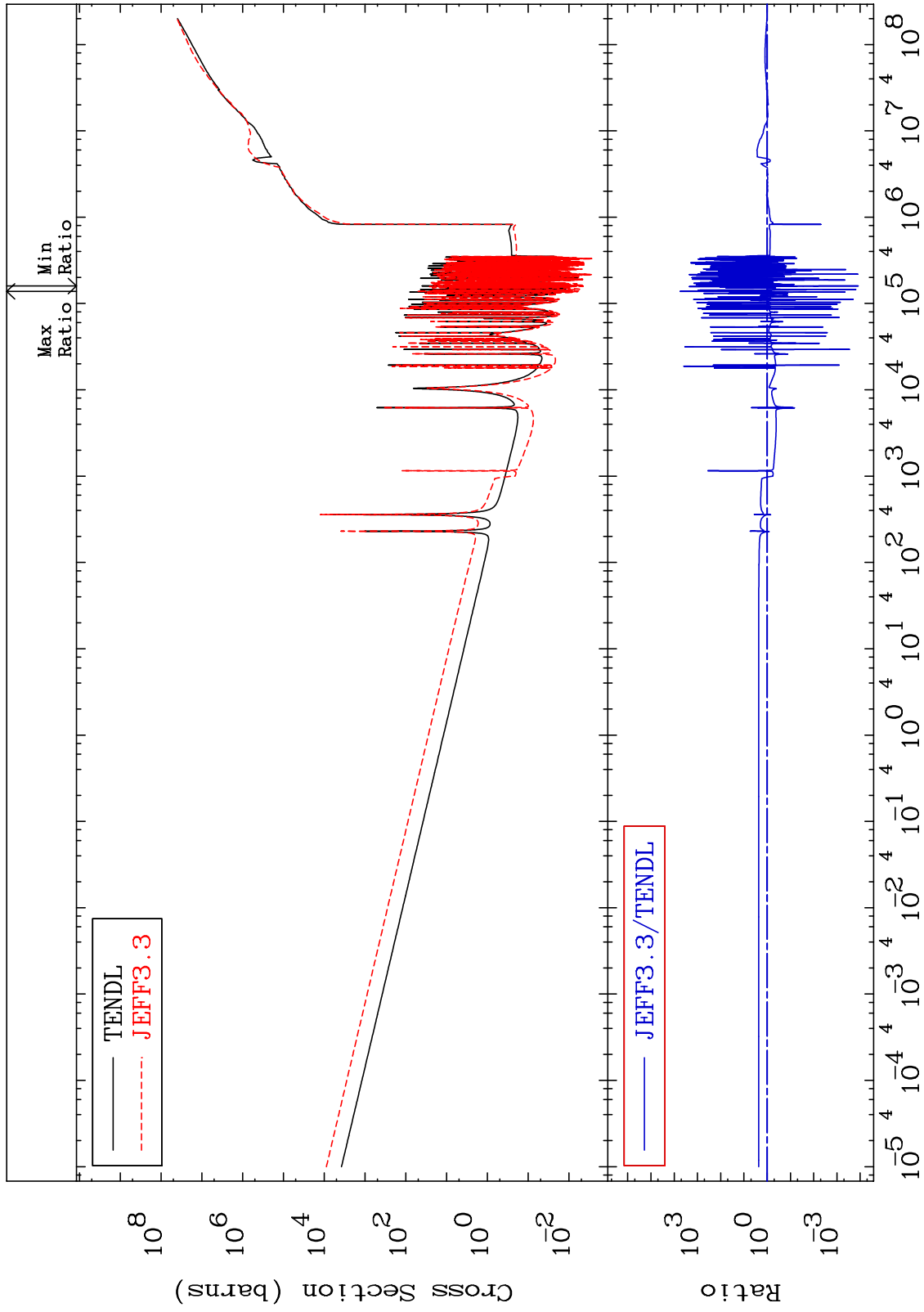
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma non-elastic (all but mt2)
Cross Section

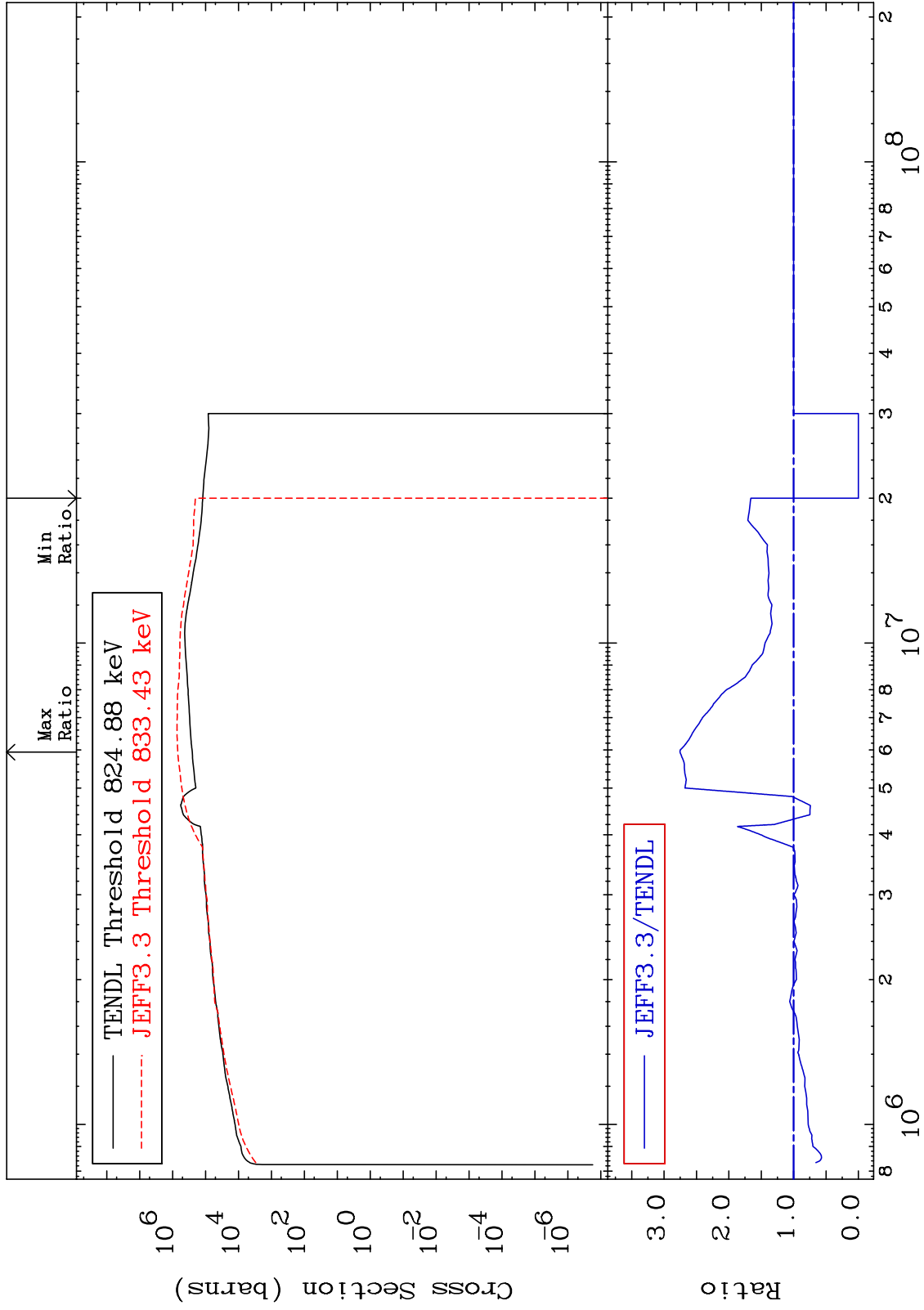
26-Fe-58
-99.99 To 9999. %



MAT 2637

Kerma inelastic (mt51-91)
Cross Section

26-Fe-58
-100.0 To 175.4 %



45

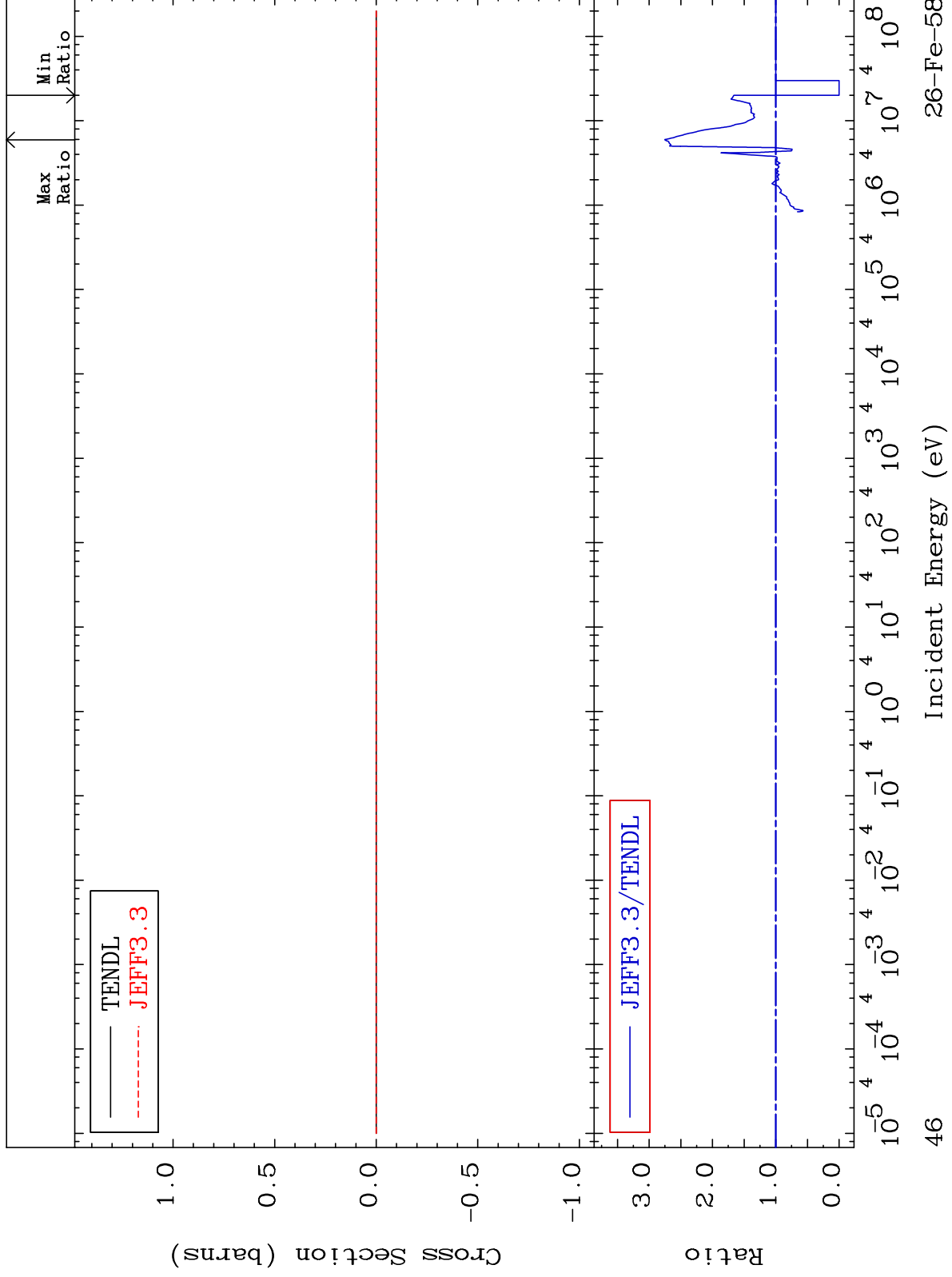
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

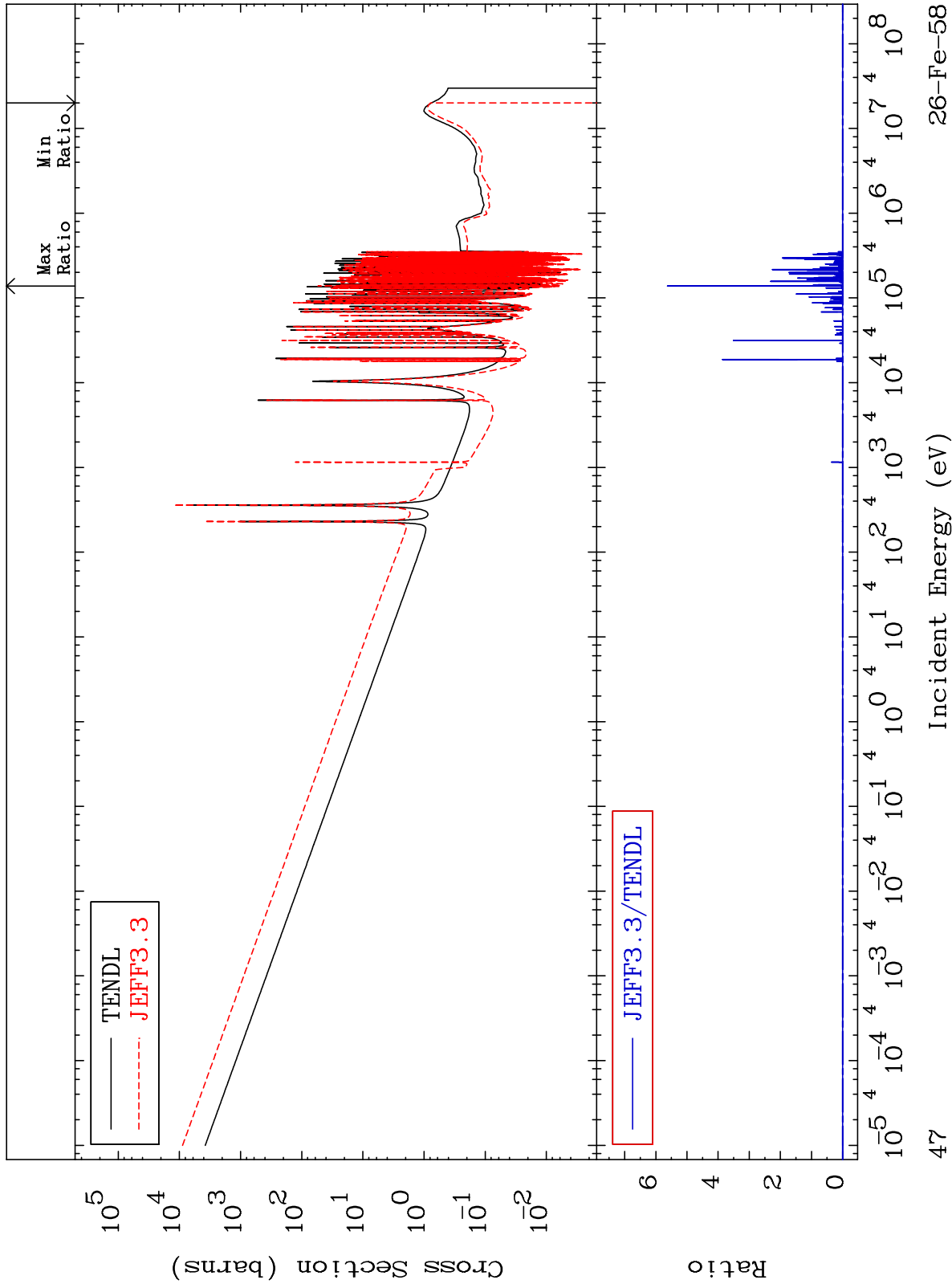
26-Fe-58
-100.0 To 175.4 %



MAT 2637

Kerma capture (mt102)
Cross Section

26-Fe-58
-100.0 To 9999. %



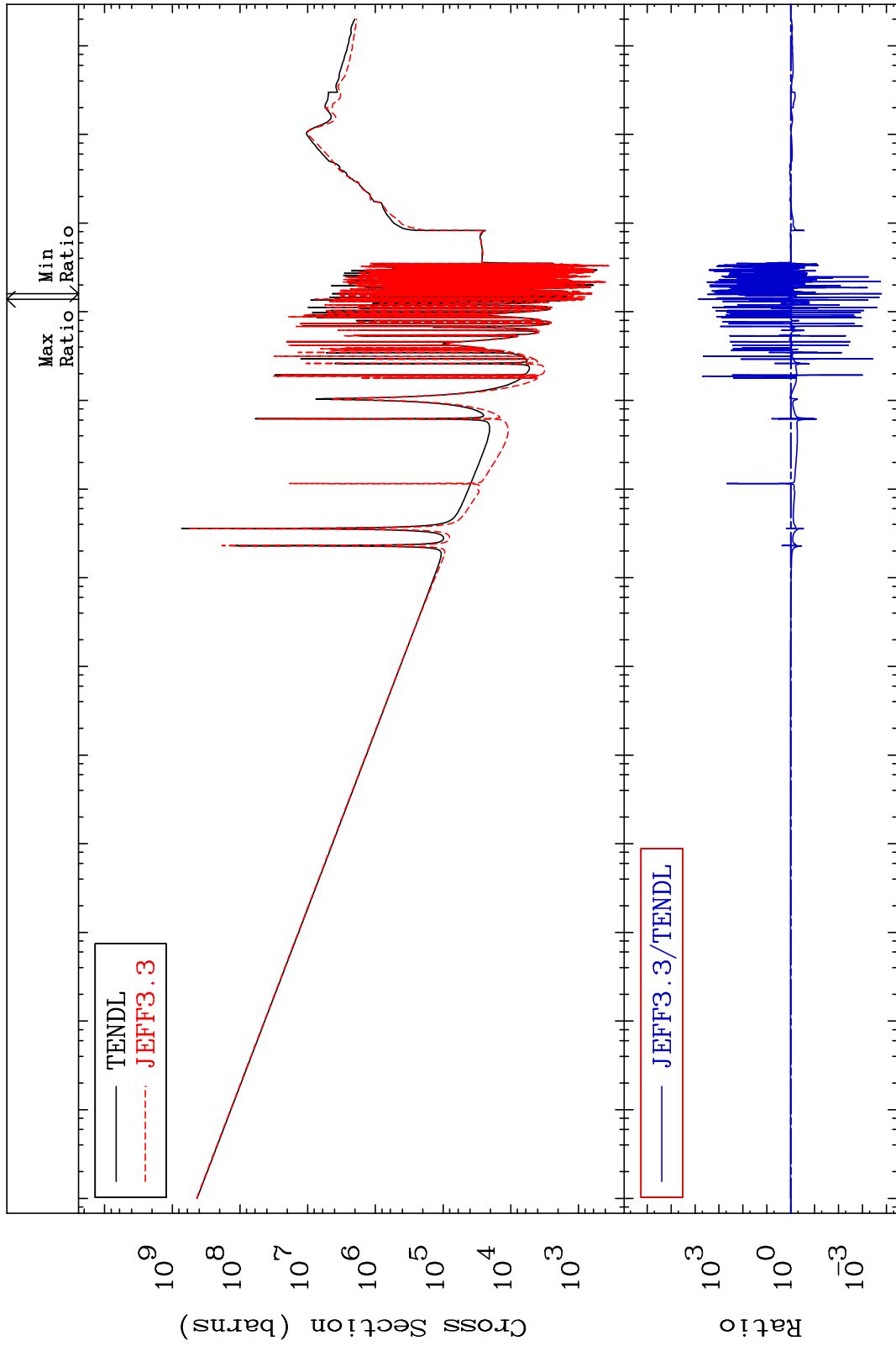
47

26-Fe-58

MAT 2637

Total photon (eV-barns)
Cross Section

26-Fe-58
-99.98 To 9999. %



48

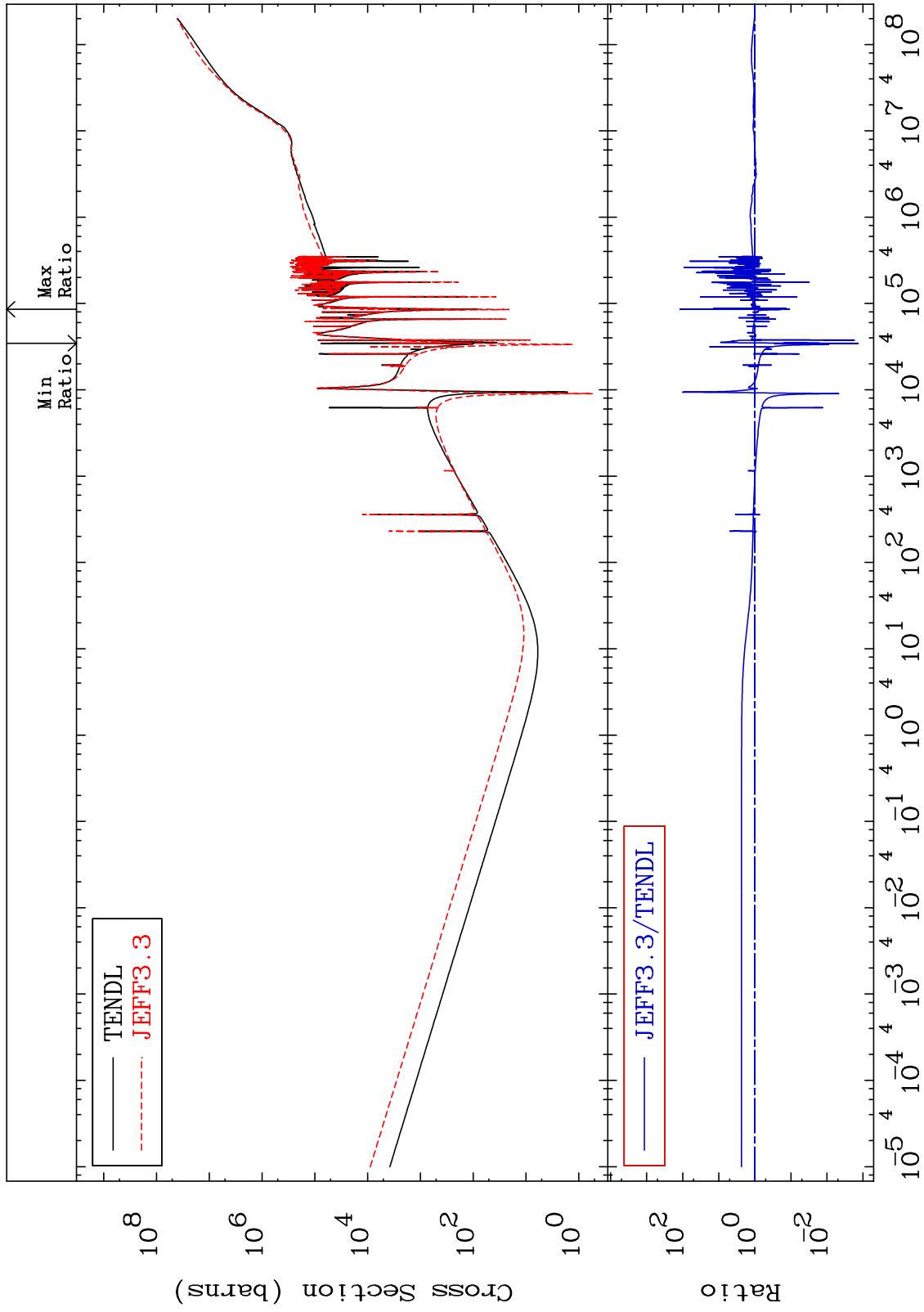
Incident Energy (eV)

26-Fe-58

MAT 2637

Total kinematic kerma (high limit)
Cross Section

26-Fe-58
-99.87 To 9999. %



49

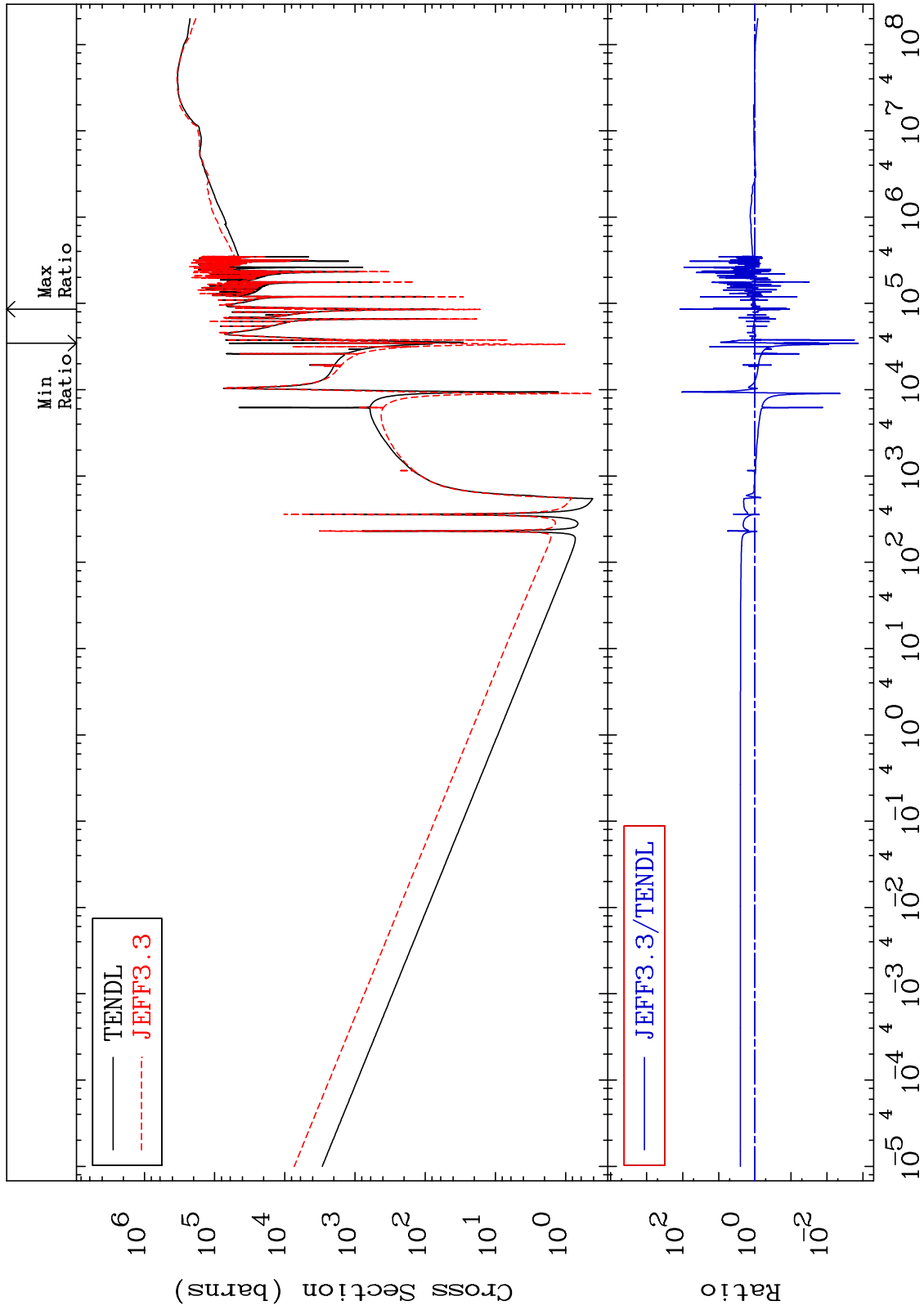
Incident Energy (eV)

26-Fe-58

MAT 2637

Dpa total (eV-barns)
Cross Section

26-Fe-58
-99.87 To 9999. %



50

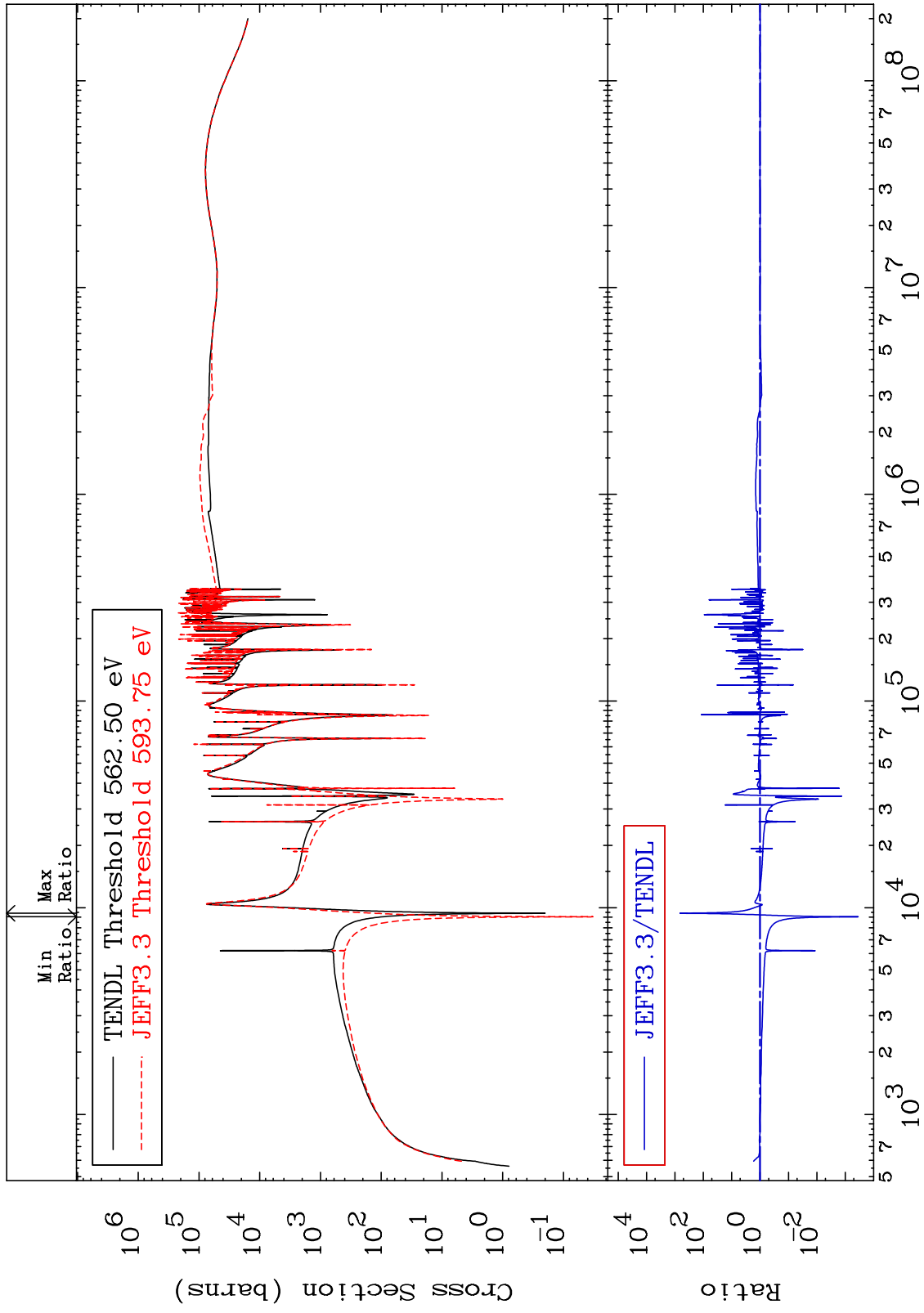
Incident Energy (eV)

26-Fe-58

MAT 2637

Dpa elastic (mt2)
Cross Section

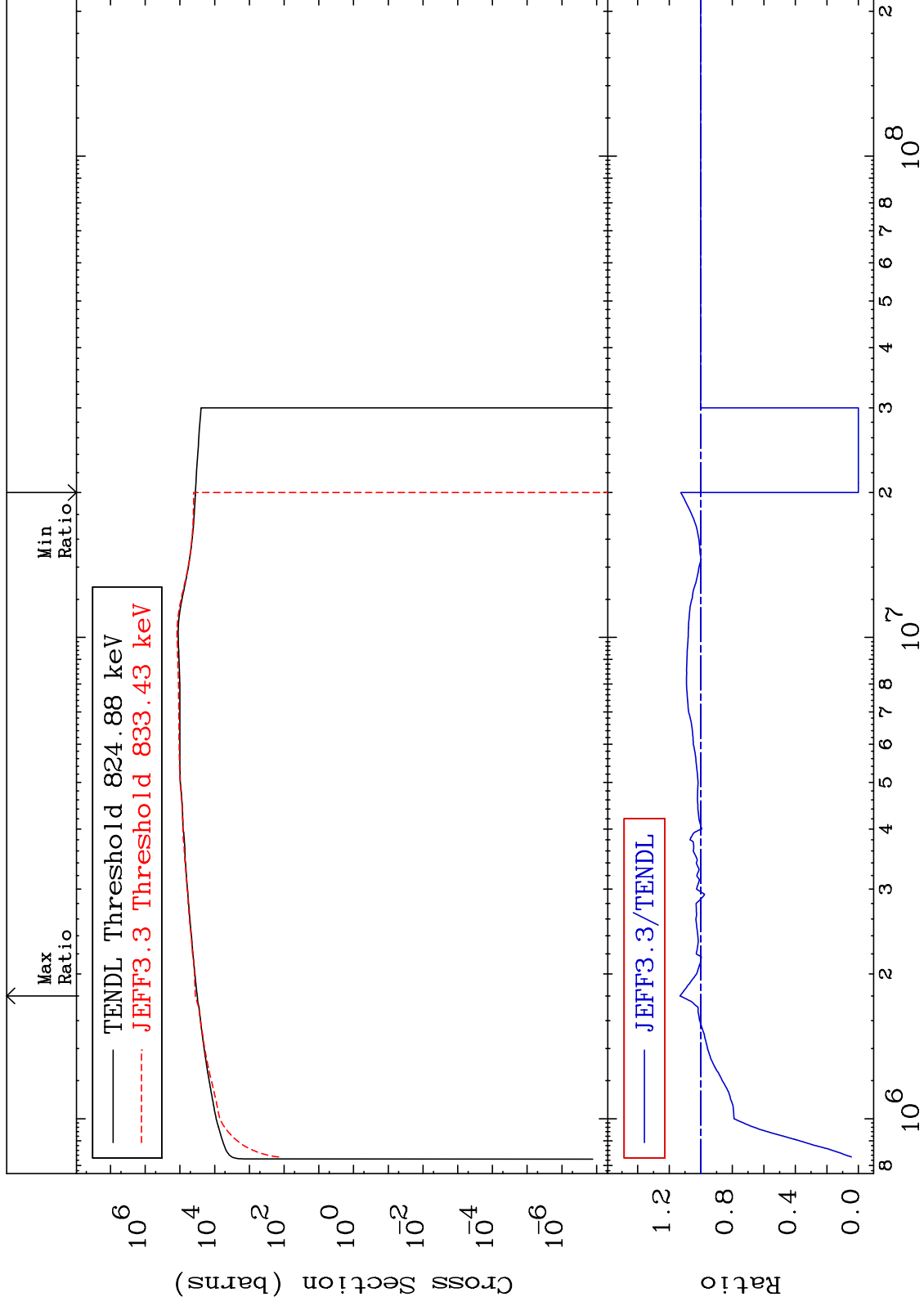
26-Fe-58
-99.97 To 9999. %



MAT 2637

Dpa inelastic (mt51-91)
Cross Section

²⁶Fe-58
-100.0 To 13.16 %



52

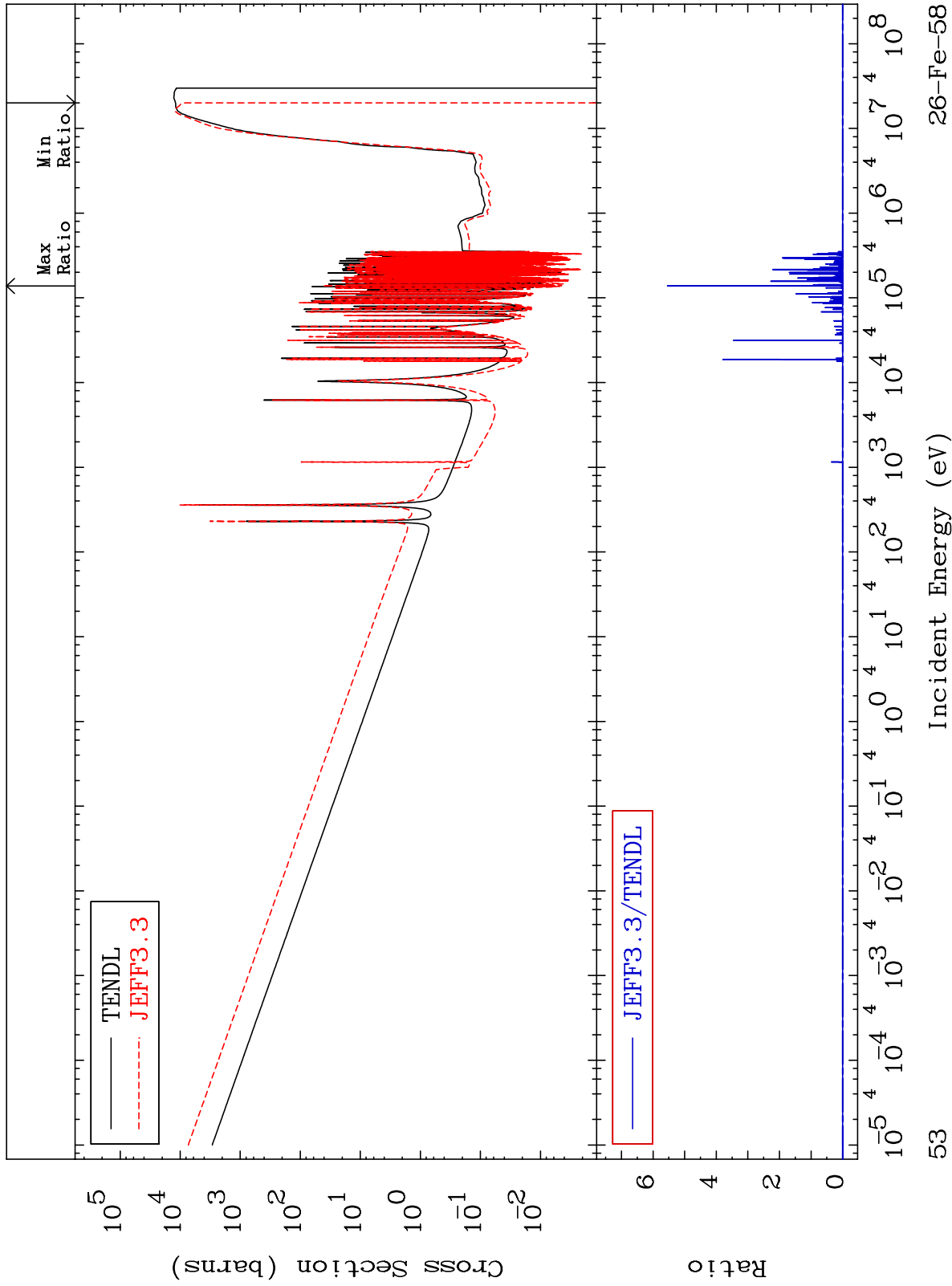
Incident Energy (eV)

²⁶Fe-58

MAT 2637

Dpa disappearance (mt102 -120)
Cross Section

26-Fe-58
-100.0 To 9999. %



53

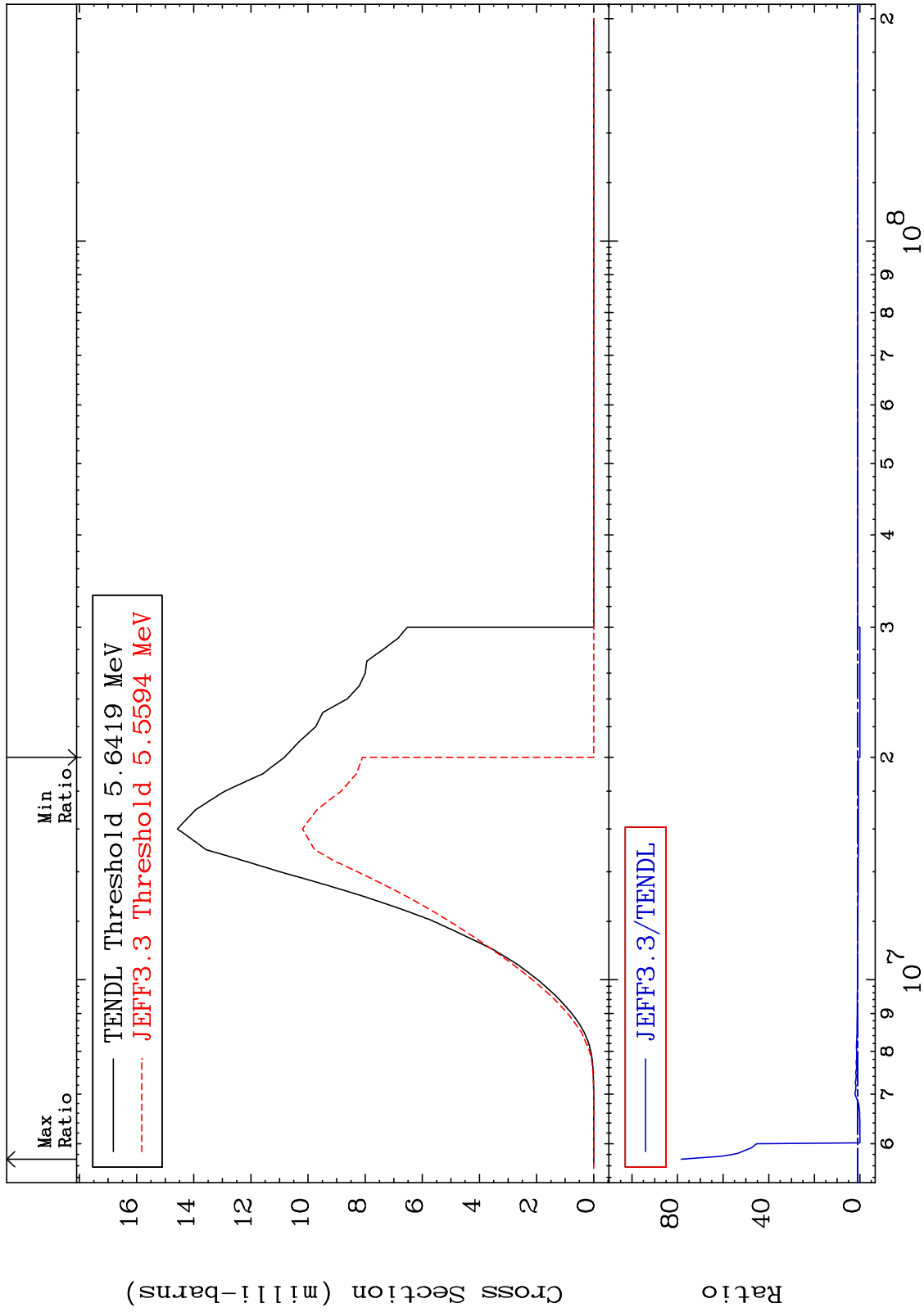
26-Fe-58

MAT 2637

(n,p):25-Mn-58g

26-Fe-58

Radionuclide Production Cross Section -100.0 To 7738. %

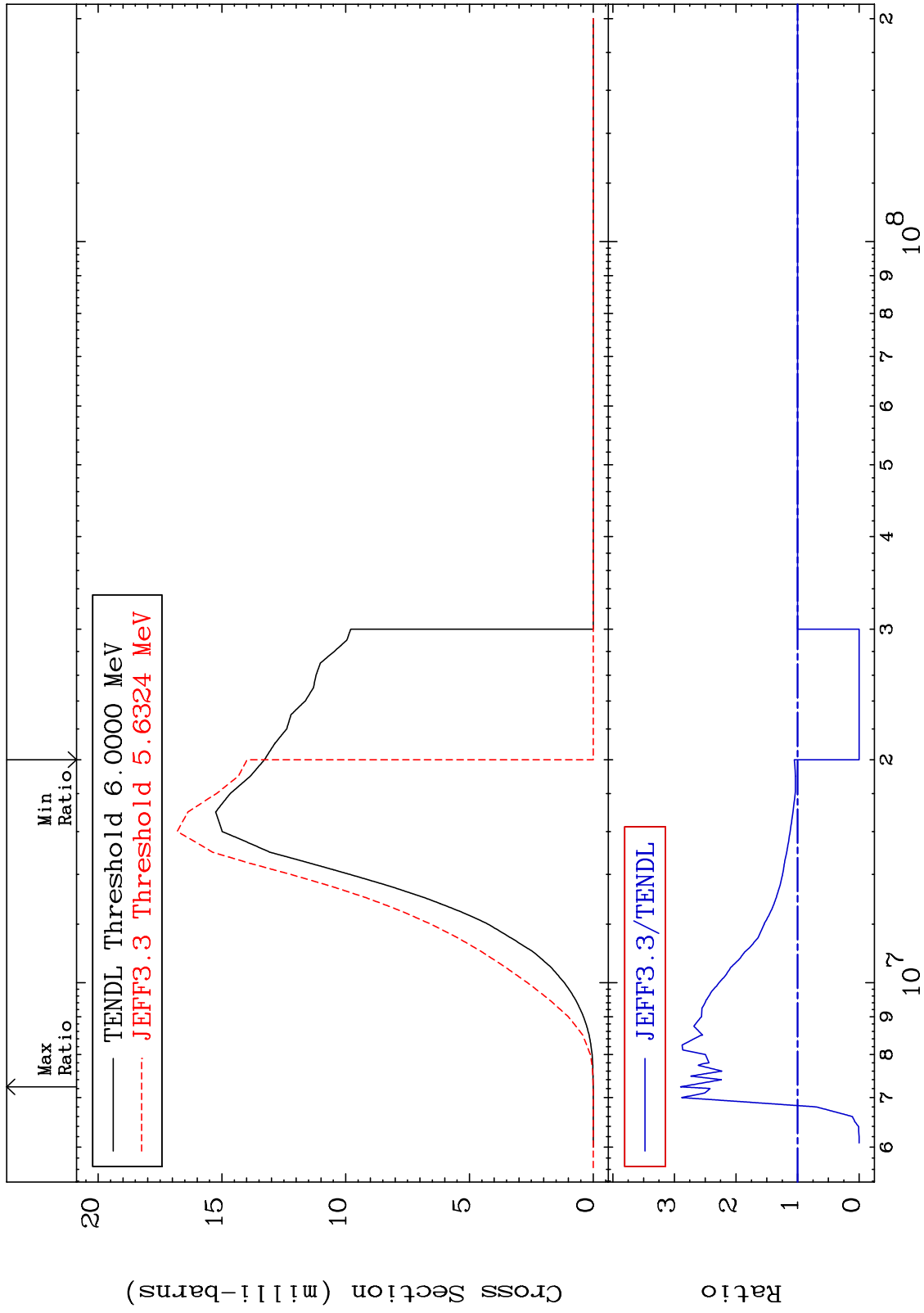


MAT 2637

(n, p):25-Mn-58m1

26-Fe-58

Radionuclide Production Cross Section -100.0 To 189.9 %



55

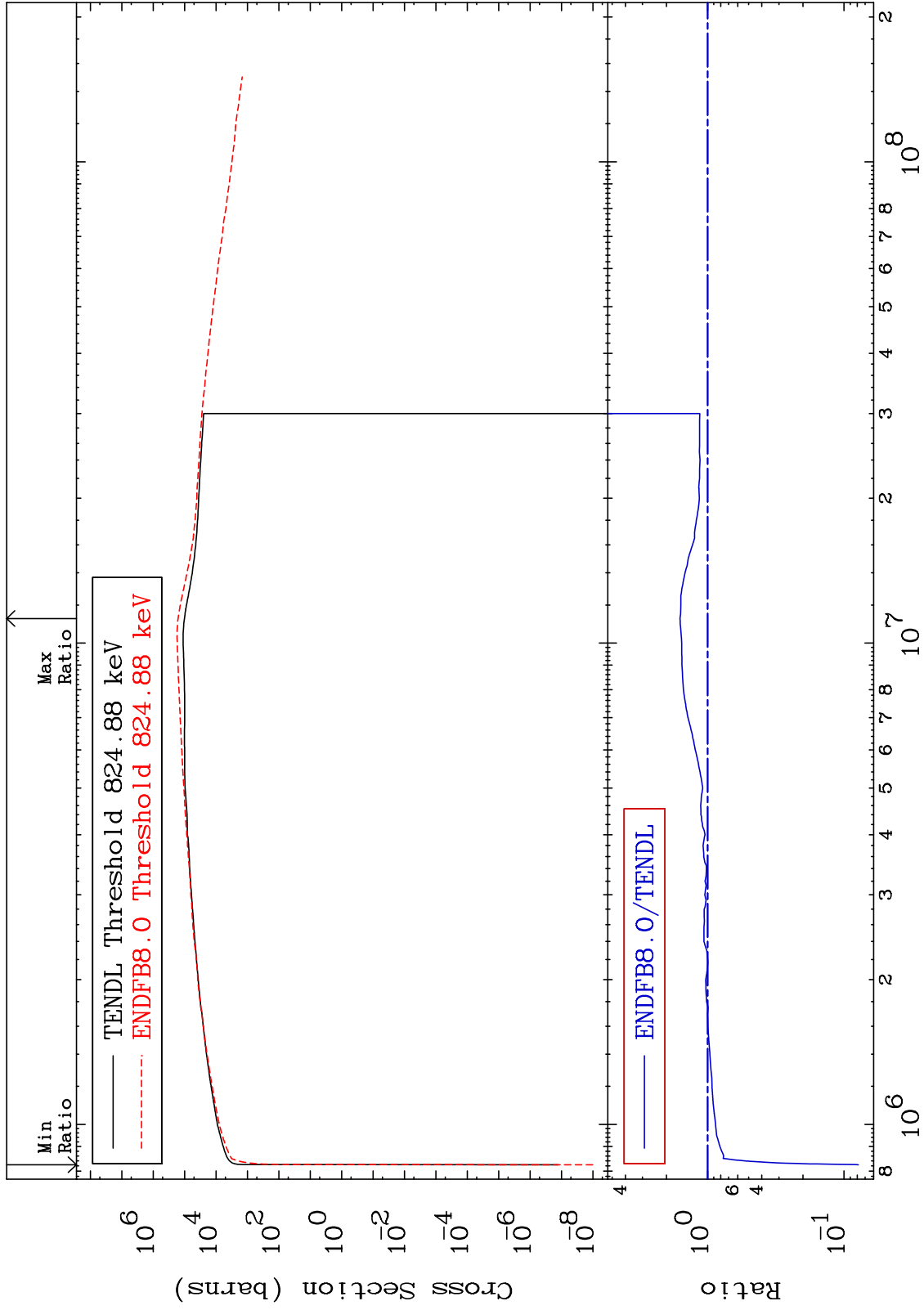
Incident Energy (eV)

26-Fe-58

MAT 2637

Dpa inelastic (mt51-91)
Cross Section

²⁶Fe-58
-92.16 To 59.19 %



56

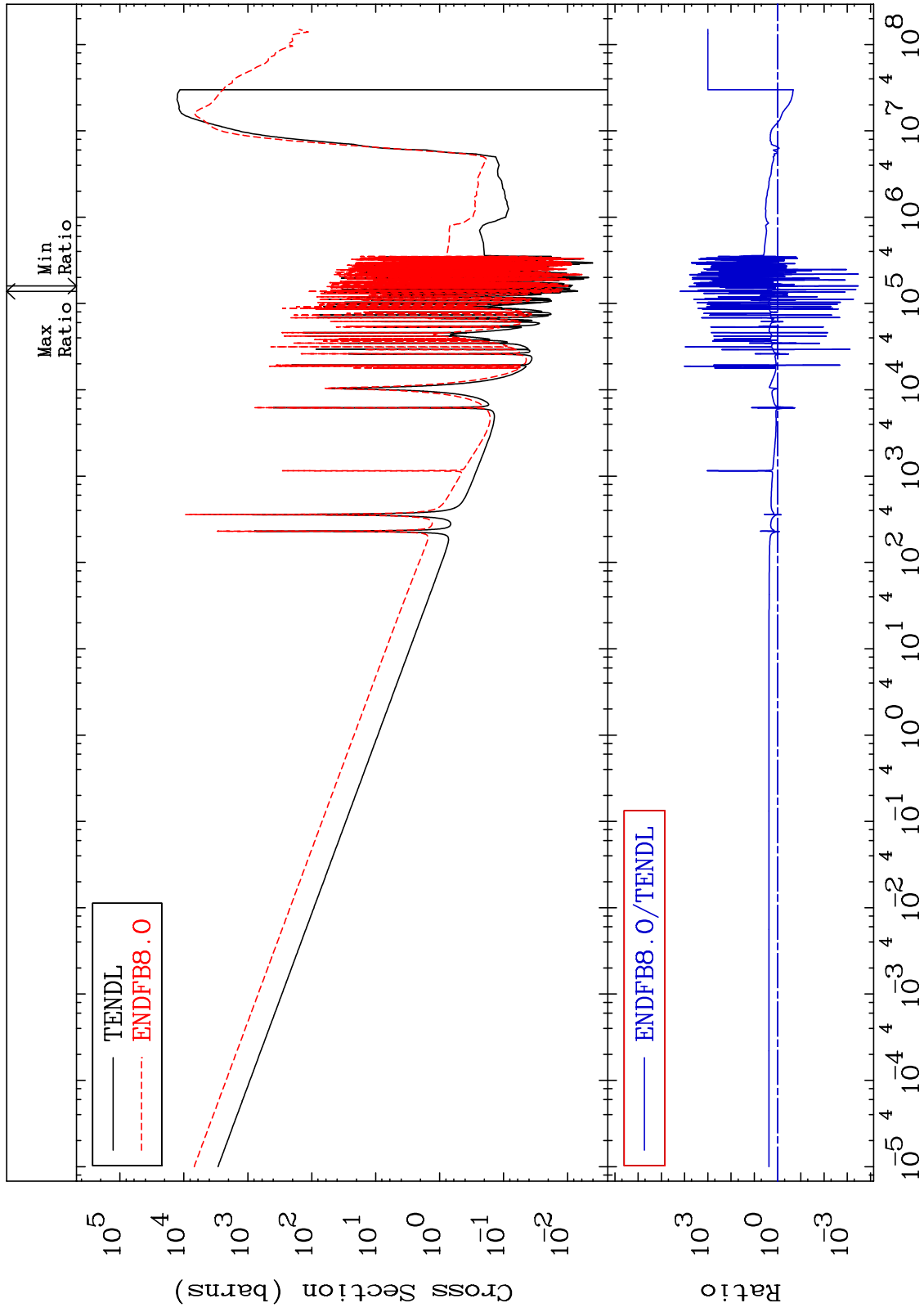
Incident Energy (eV)

²⁶Fe-58

MAT 2637

Dpa disappearance (mt102 -120)
Cross Section

26-Fe-58
-99.97 To 9999. %



57

Incident Energy (eV)

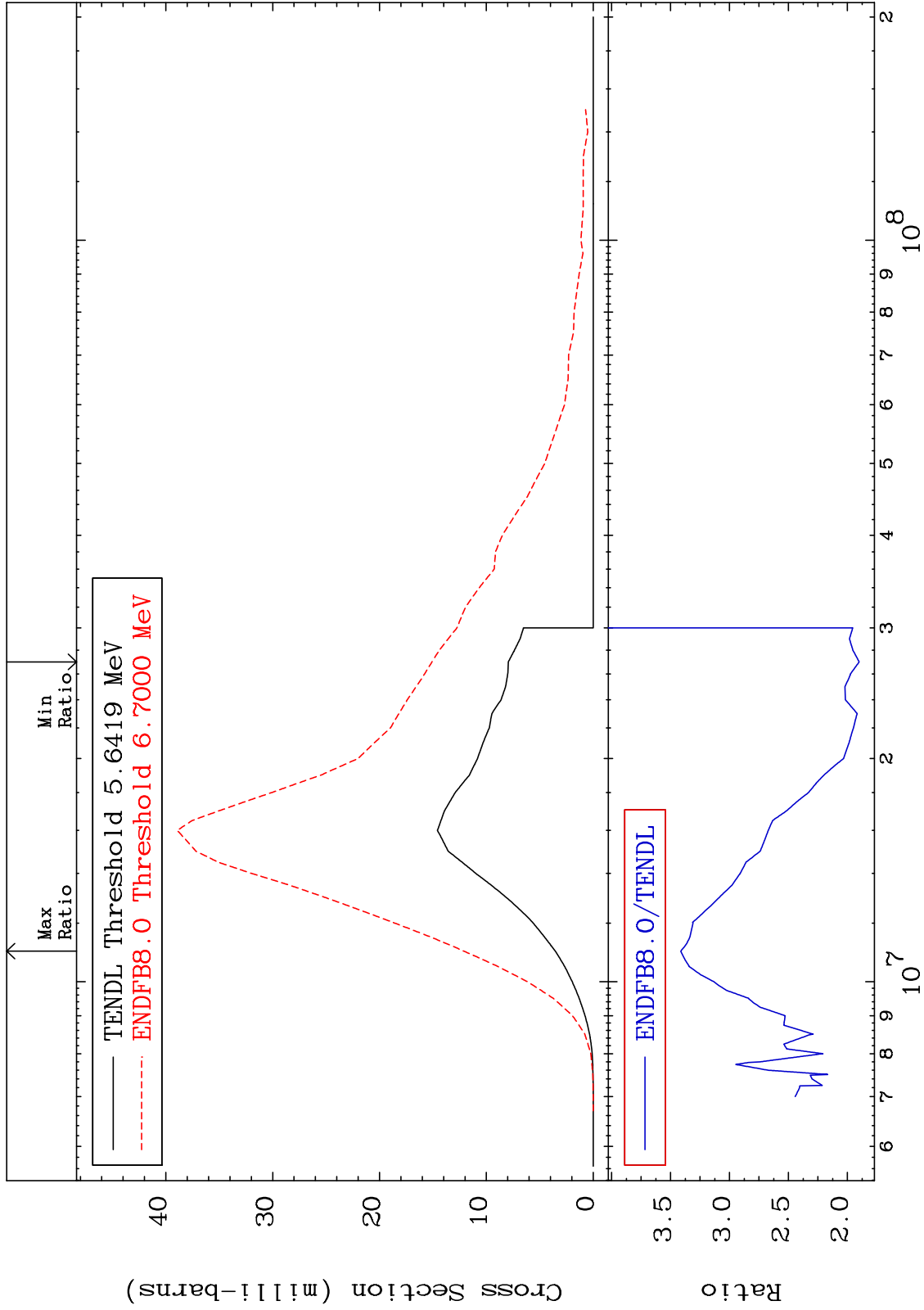
26-Fe-58

MAT 2637

(n, p) : 25-Mn-58g

26-Fe-58

Radionuclide Production Cross Section 90.01 To 241.4 %



58

Incident Energy (eV)

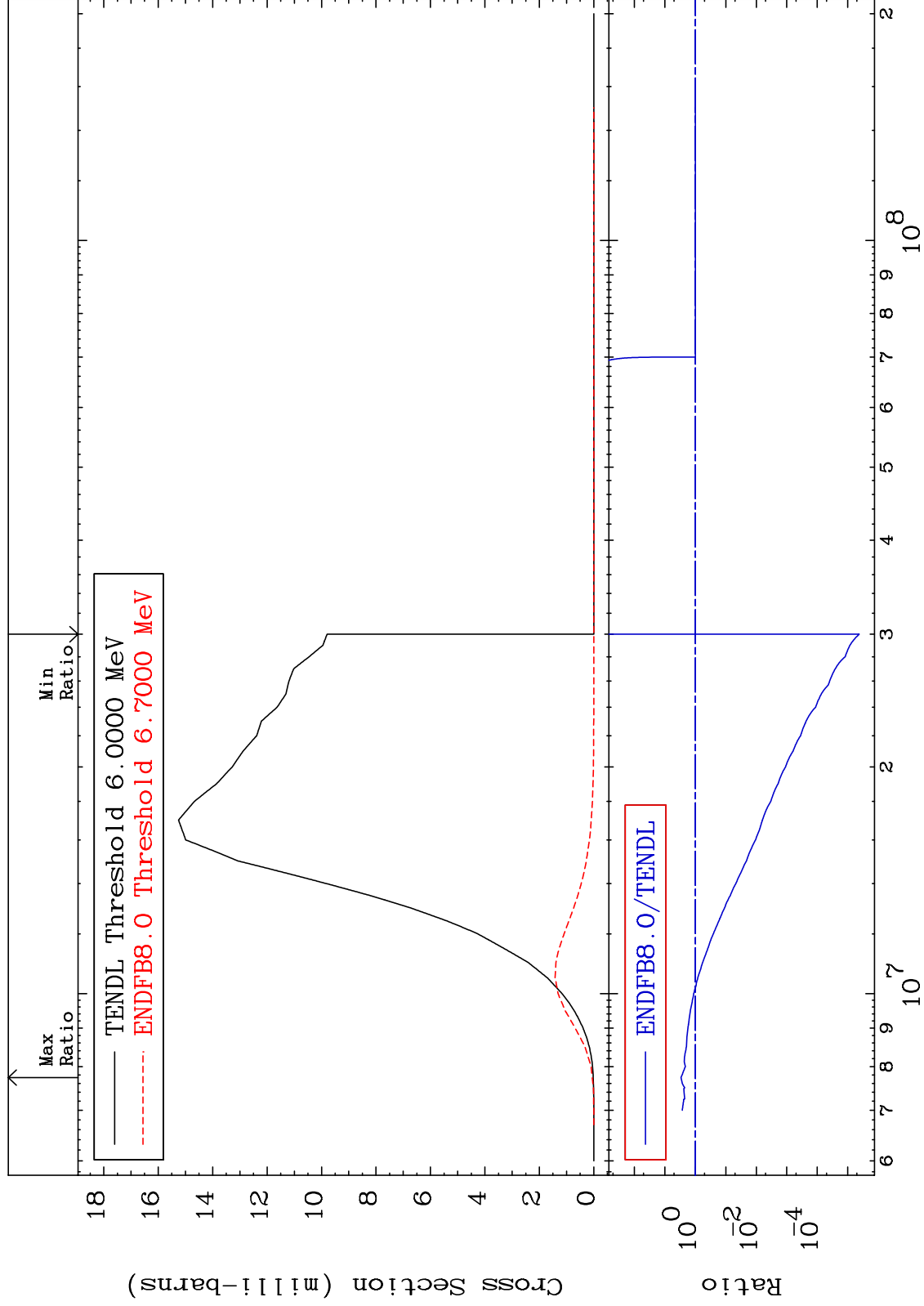
26-Fe-58

MAT 2637

(n, p):25-Mn-58m1

26-Fe-58

Radionuclide Production Cross Section -100.0 To 193.2 %



59

Incident Energy (eV)

26-Fe-58