

Program Complot
(Version 2018-1)

by

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Press Mouse Button to Start

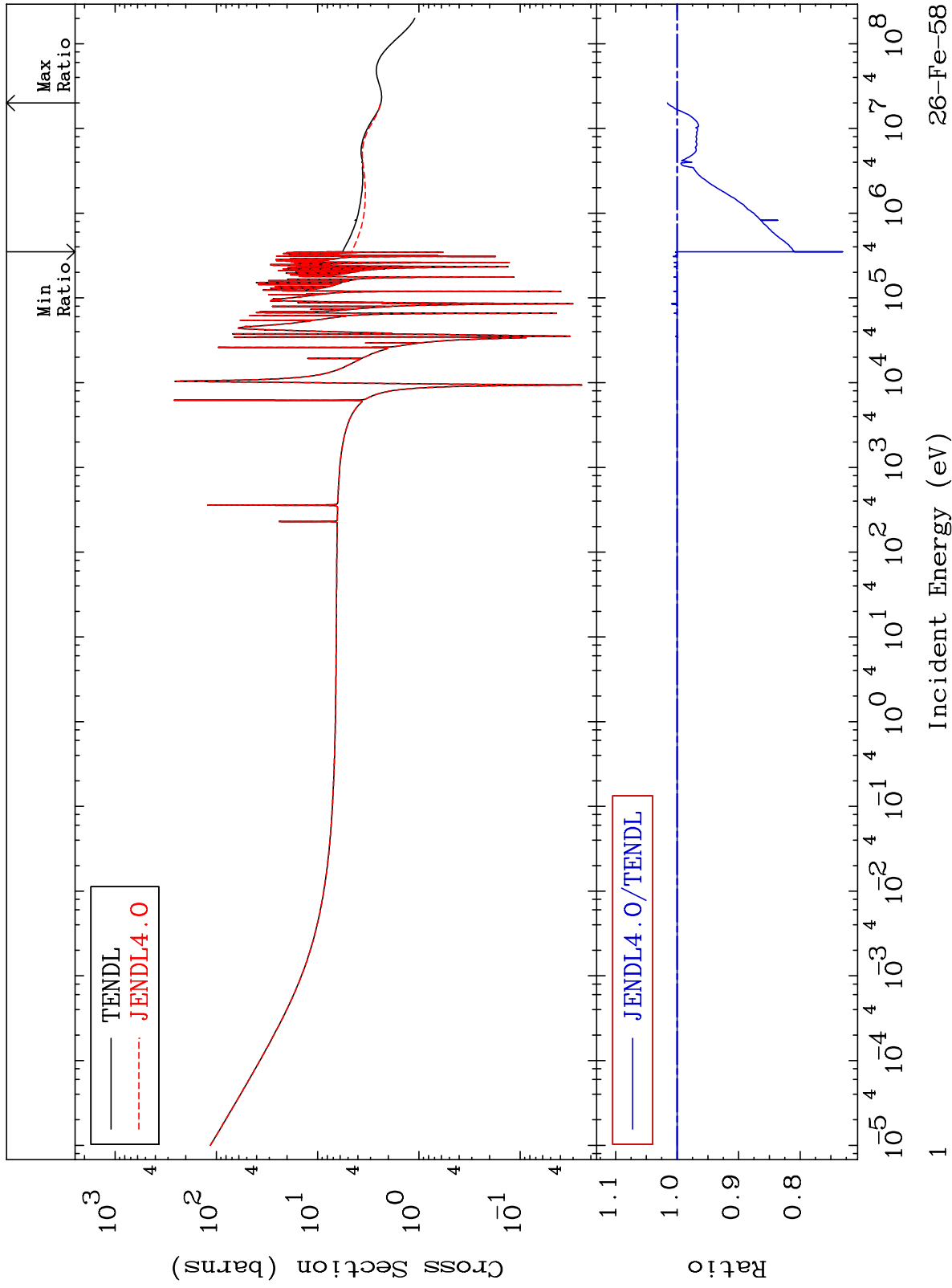
MAT 2637

Total

26-Fe-58

Cross Section

-26.92 To 1.559 %

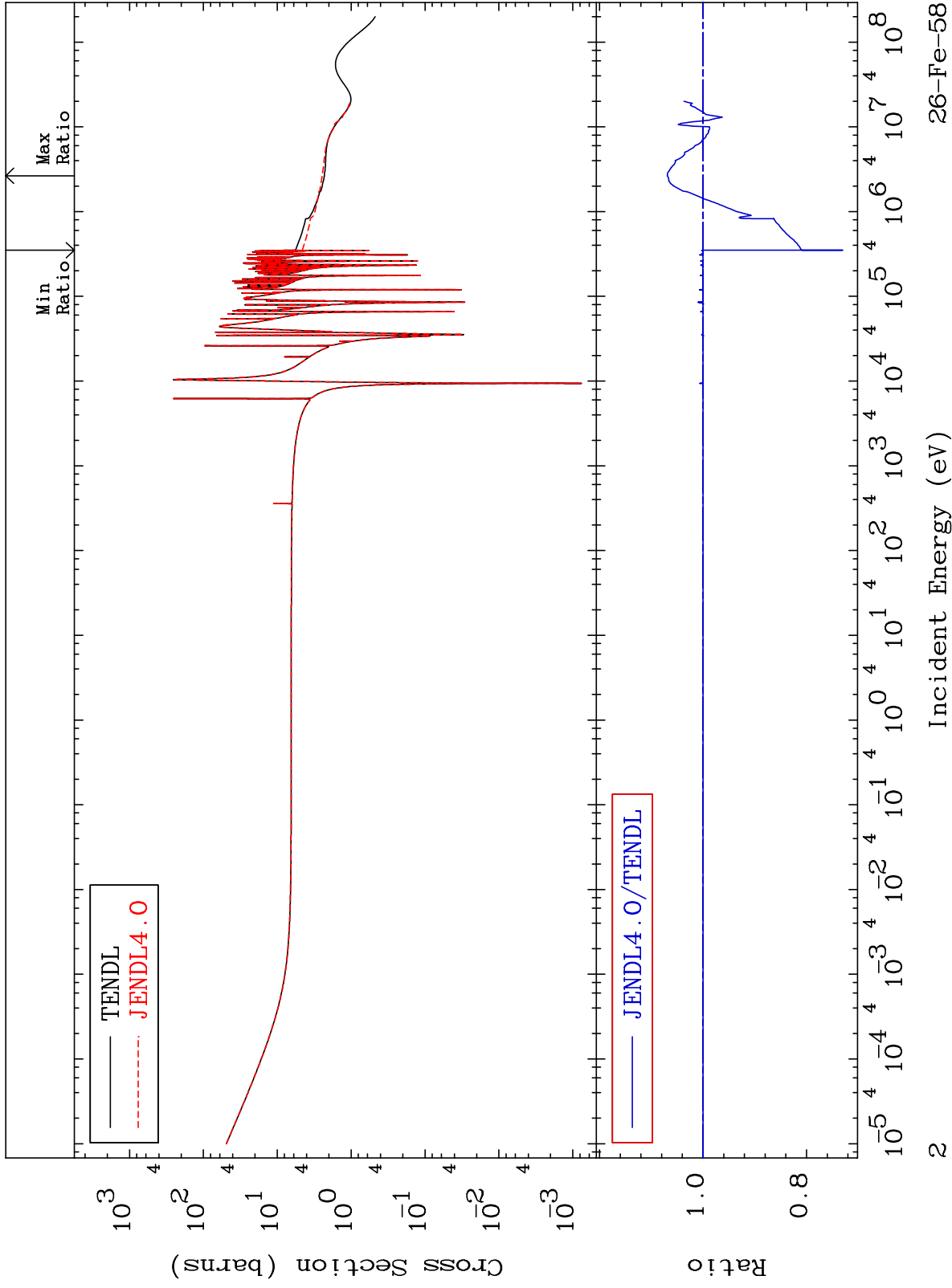


26-Fe-58

MAT 2637

Elastic
Cross Section

26-Fe-58
-26.96 To 6.900 %



26-Fe-58

Incident Energy (eV)

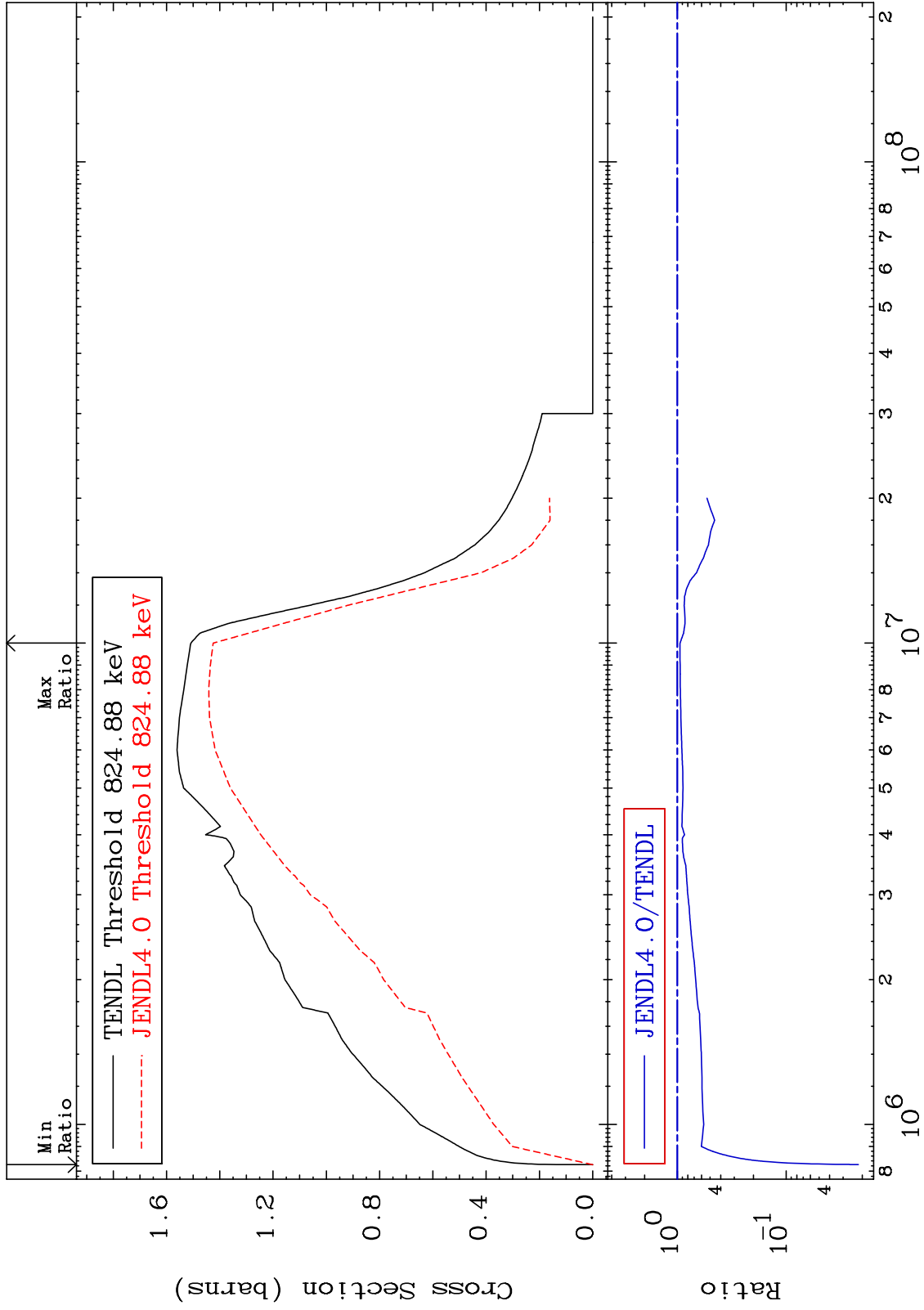
2

MAT 2637

Inelastic
Cross Section

²⁶Fe-58

-97.83 To -5.574%



Incident Energy (eV)

²⁶Fe-58

3

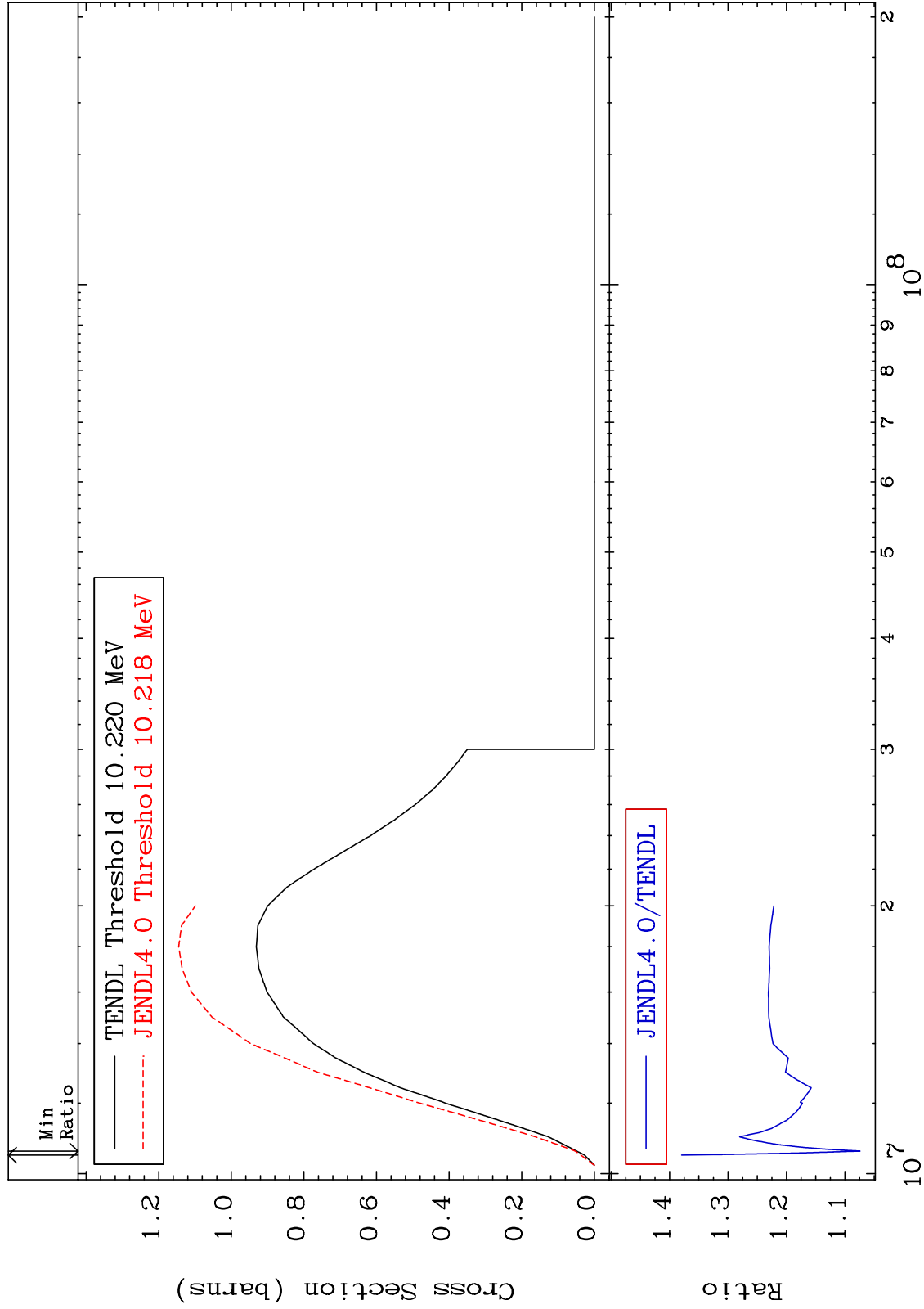
MAT 2637

(n,2n)

26-Fe-58

Cross Section

7.428 To 37.94 %



26-Fe-58

7.428 To 37.94 %

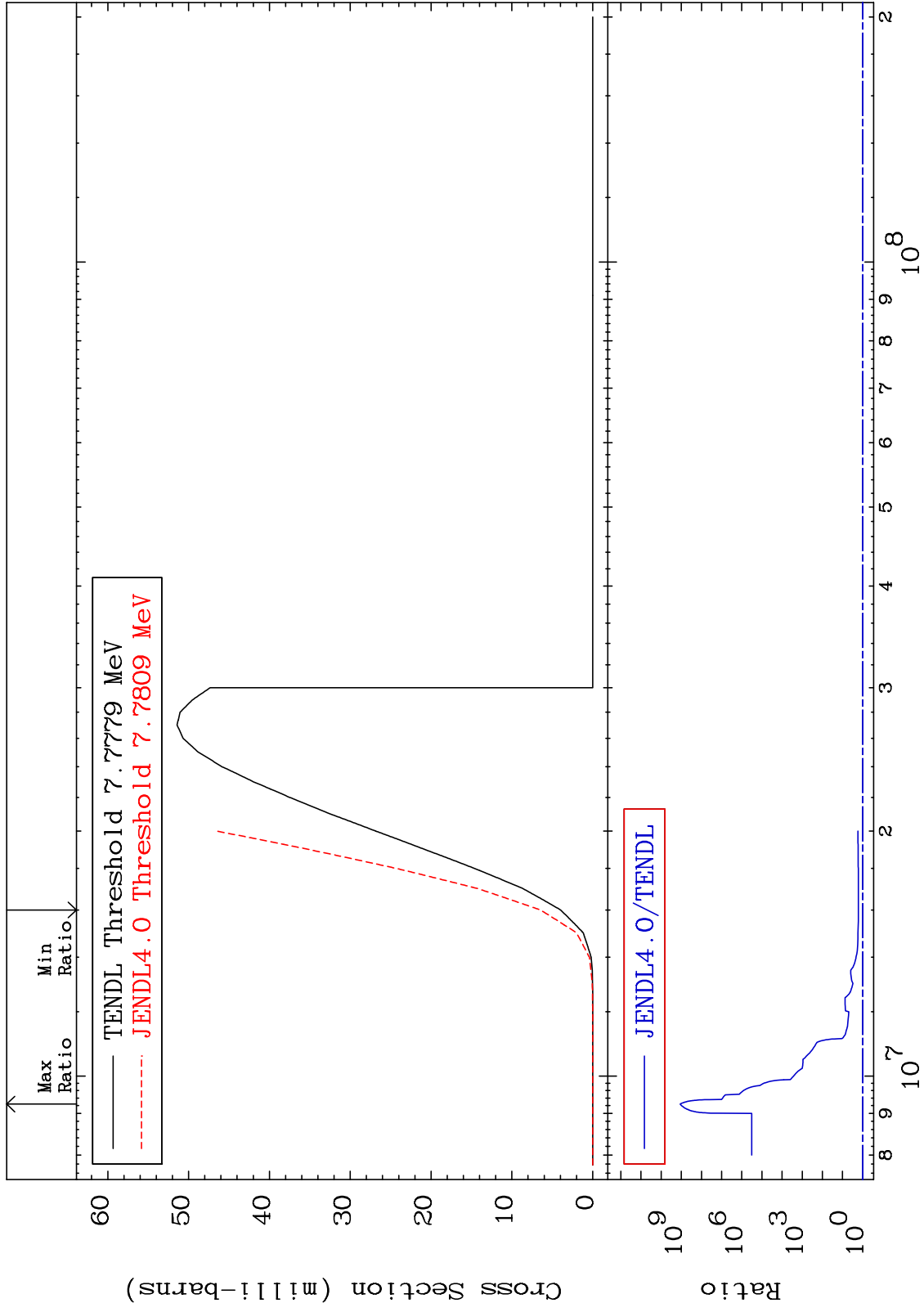
MAT 2637

(n,n') α

²⁶Fe-58

Cross Section

63.33 To 9999. %



5

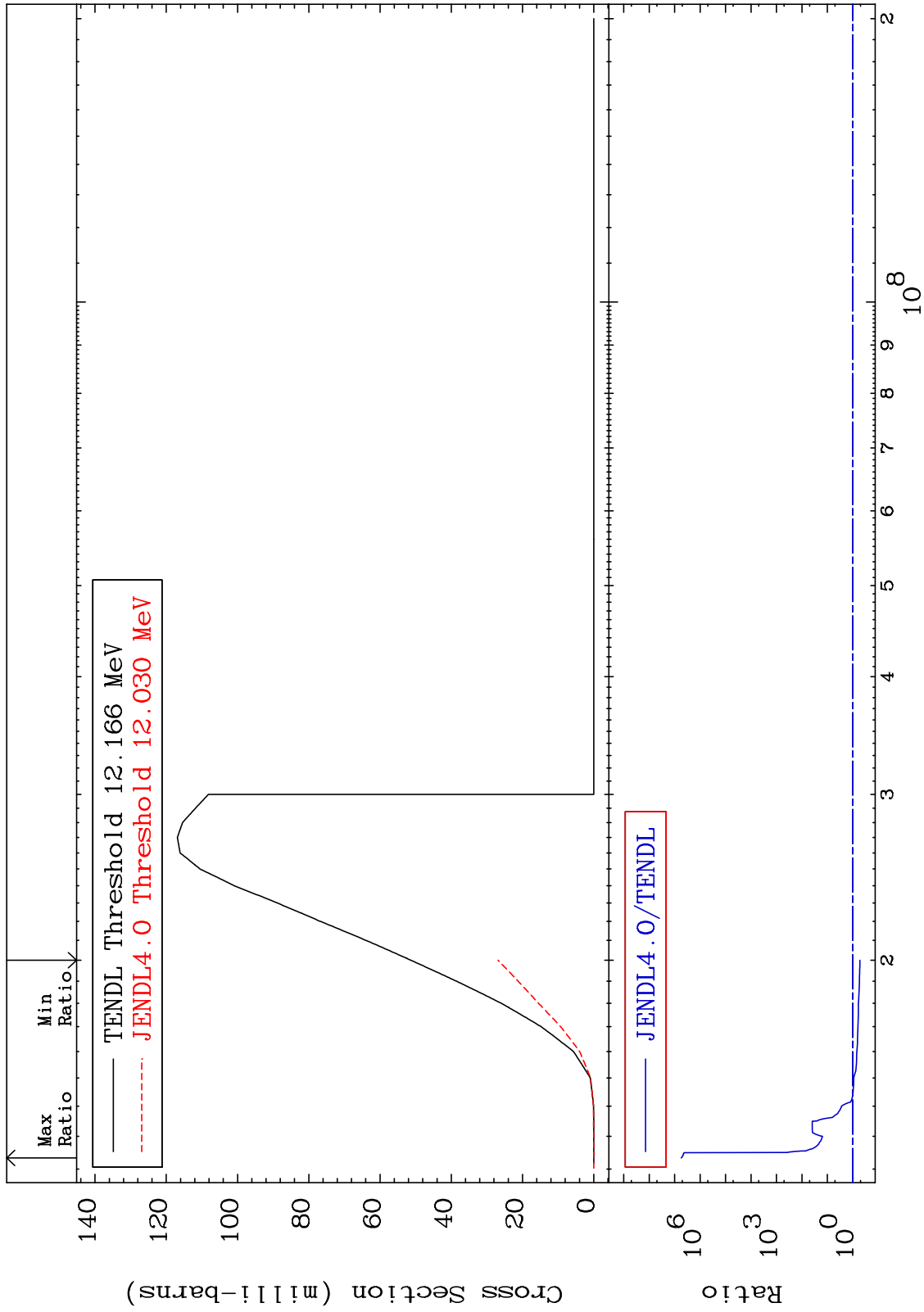
Incident Energy (eV)

²⁶Fe-58

MAT 2637

(n,n') p
Cross Section

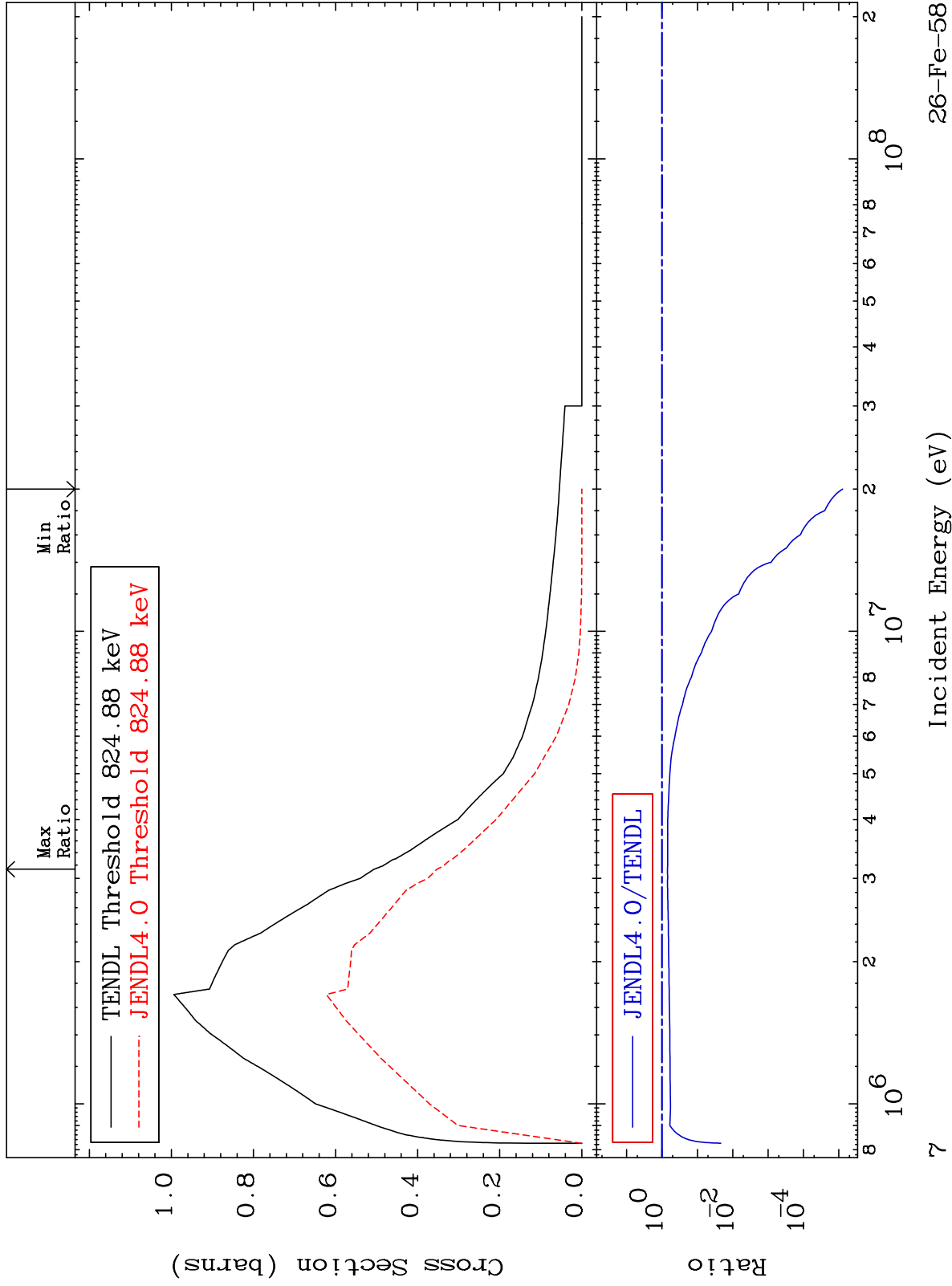
26-Fe-58
-47.65 To 9999. %



MAT 2637

MT= 51 (n,n') Level
Cross Section

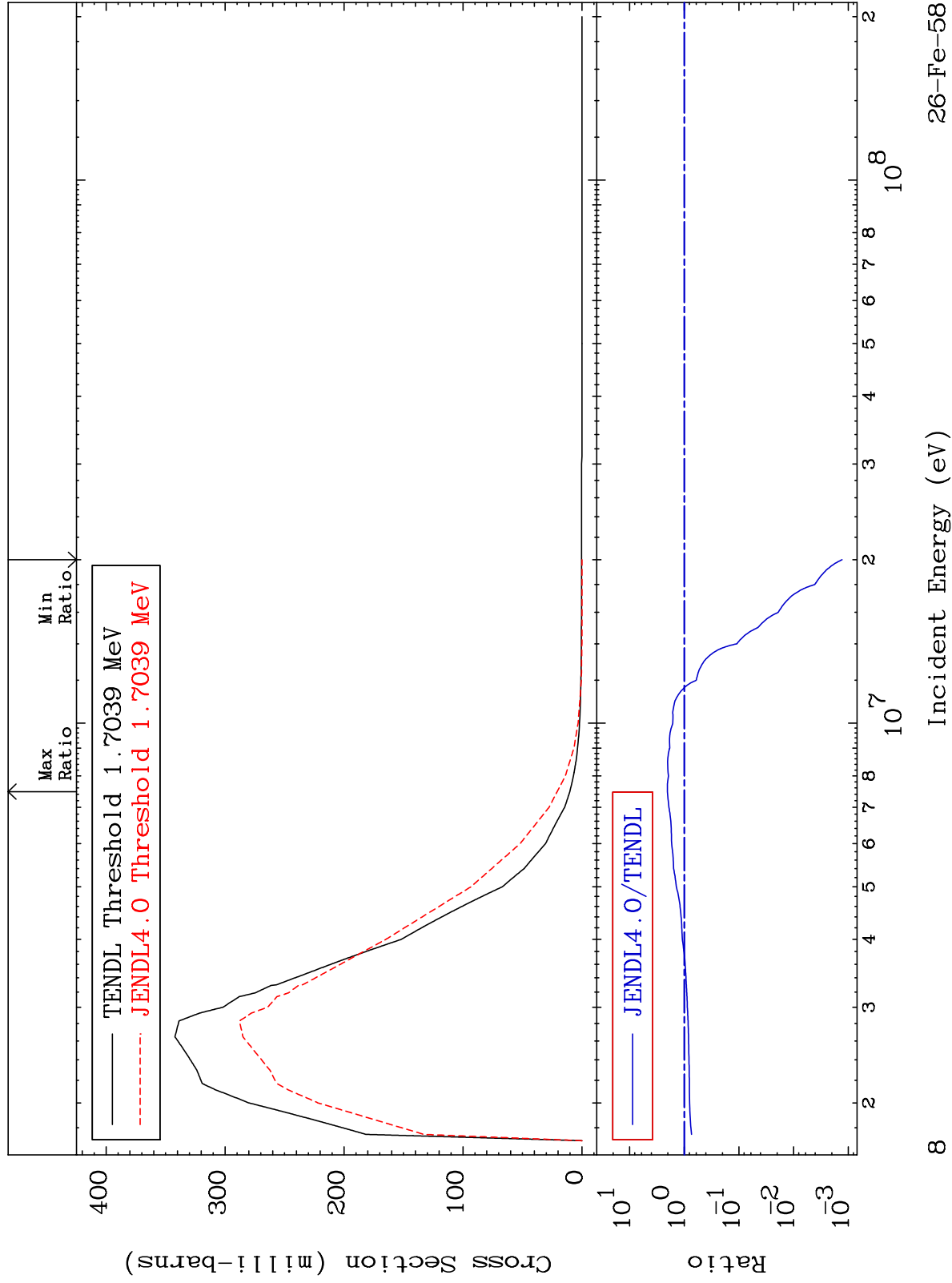
26-Fe-58
-100.0 To -30.29%



MAT 2637

MT= 52 (n,n') Level
Cross Section

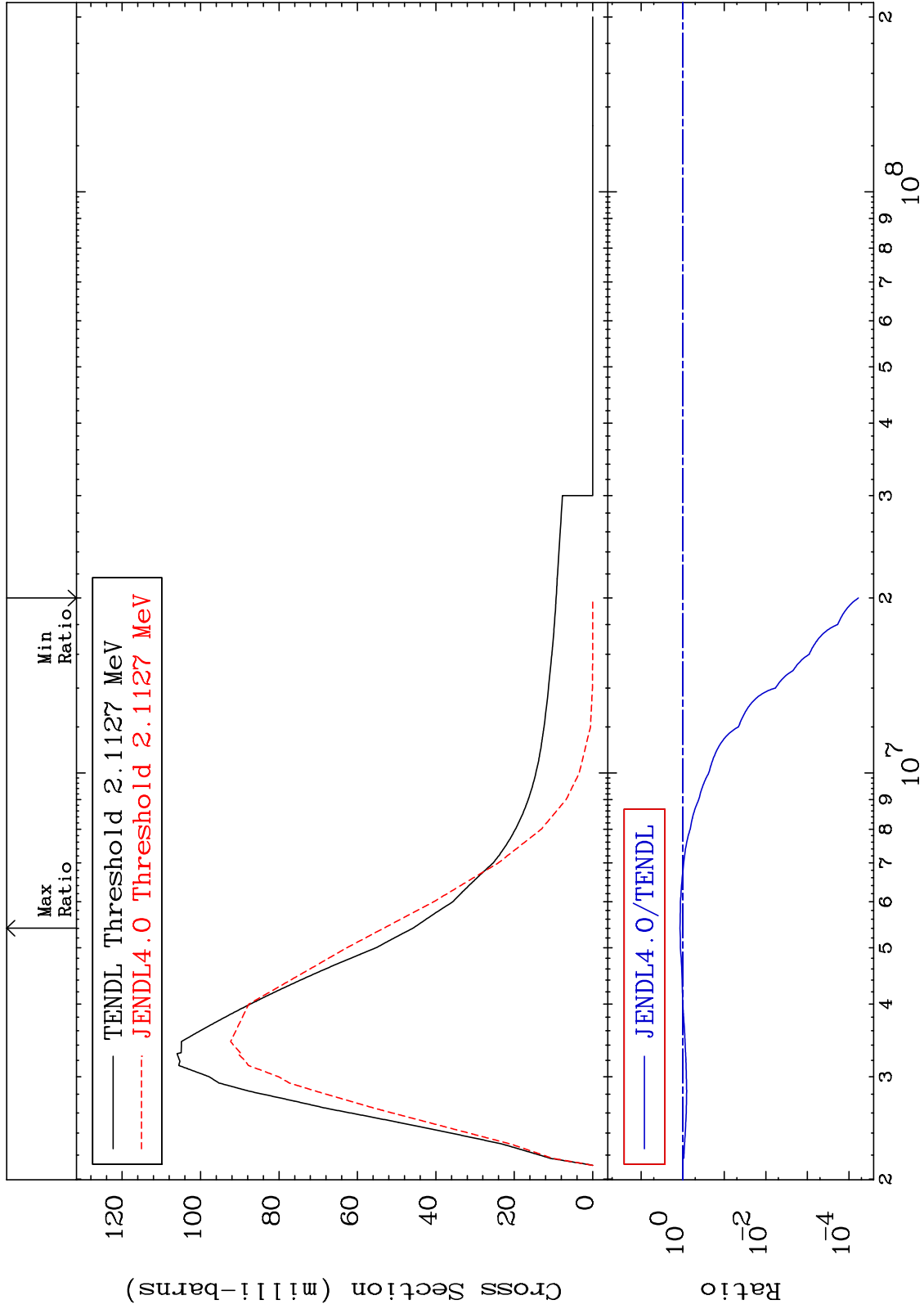
26-Fe-58
-99.87 To 102.3 %



MAT 2637

MT= 53 (n,n') Level
Cross Section

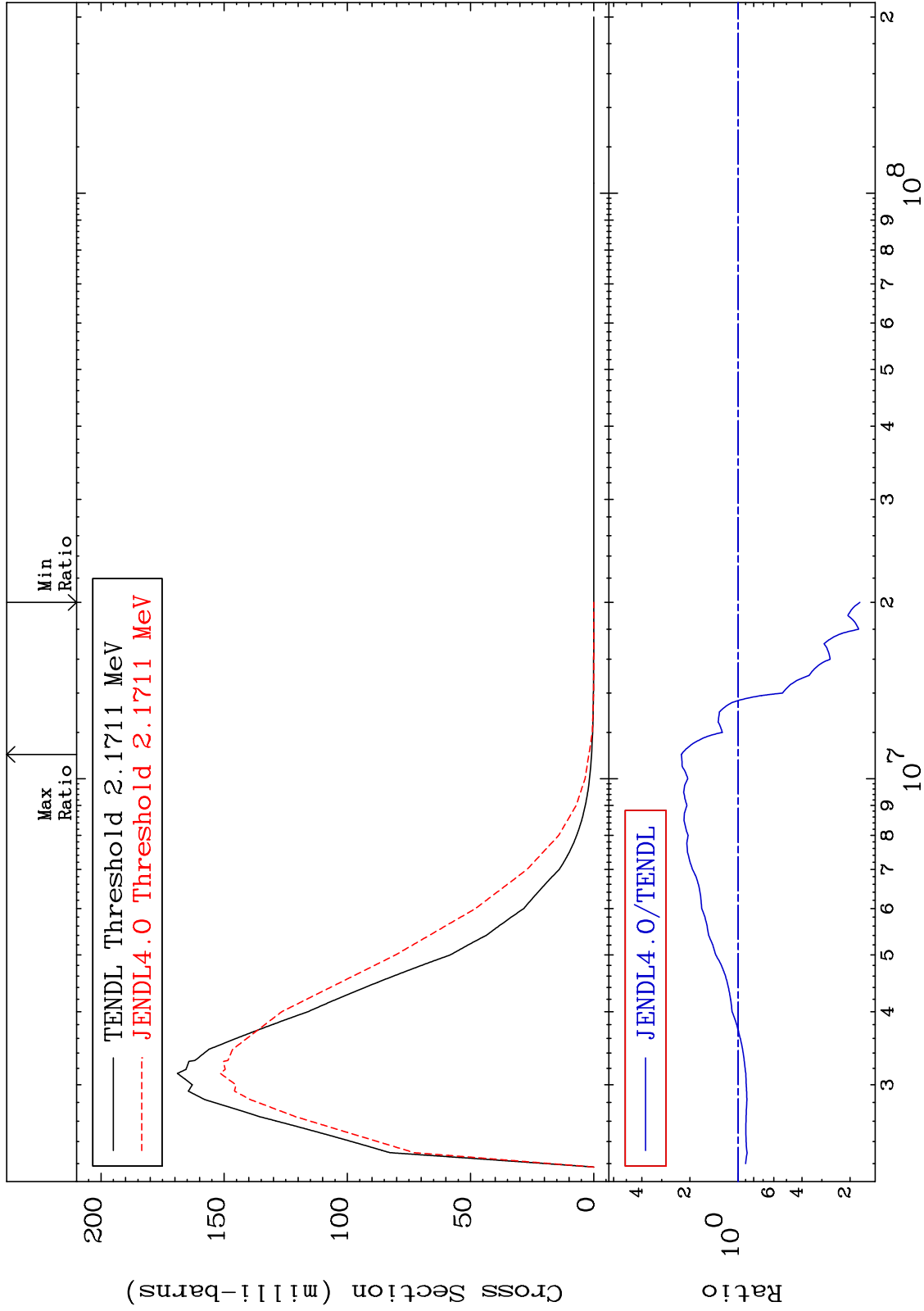
26-Fe-58
-99.99 To 16.27 %



MAT 2637

MT= 54 (n, n') Level
Cross Section

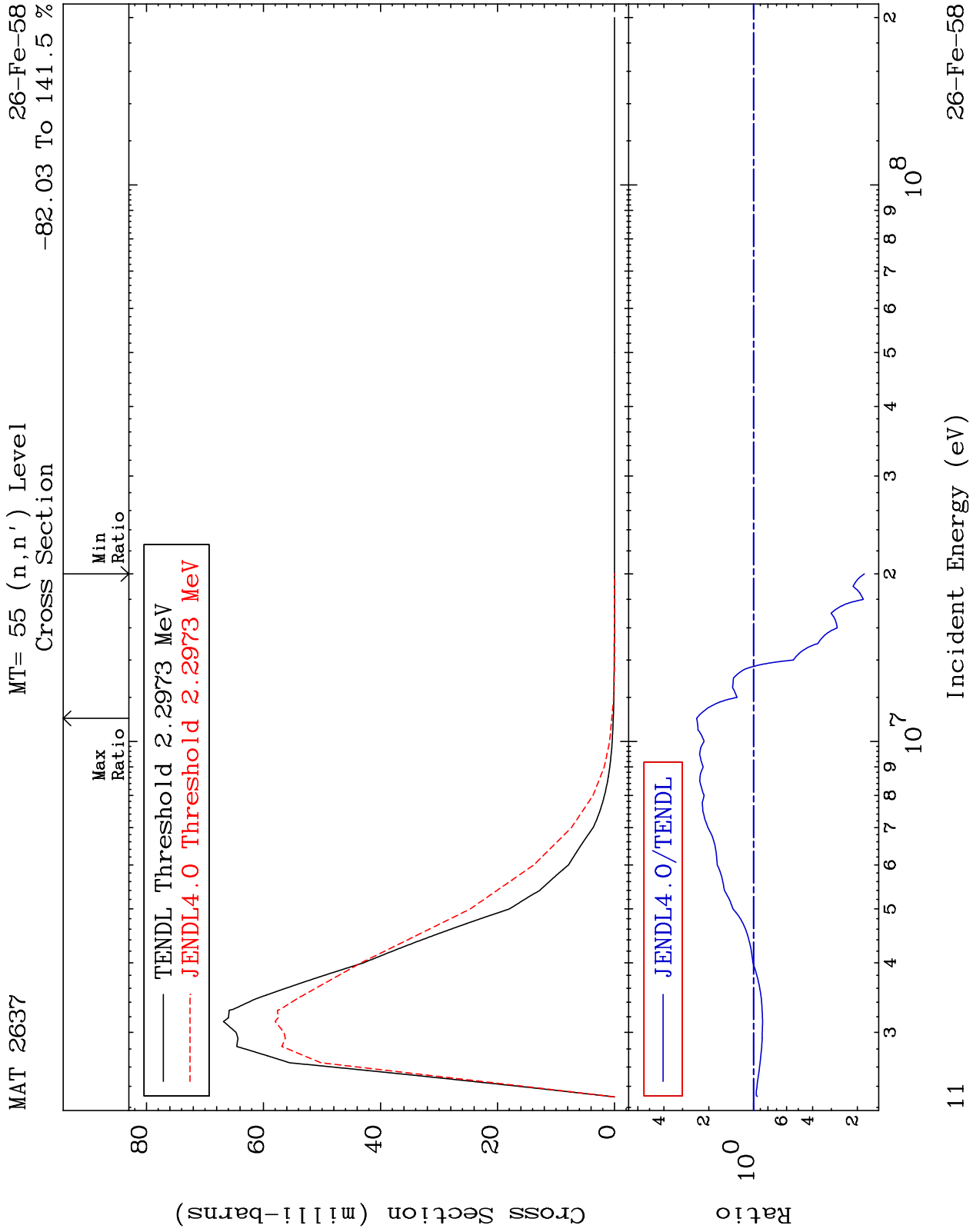
26-Fe-58
-82.63 To 126.8 %



10

Incident Energy (eV)

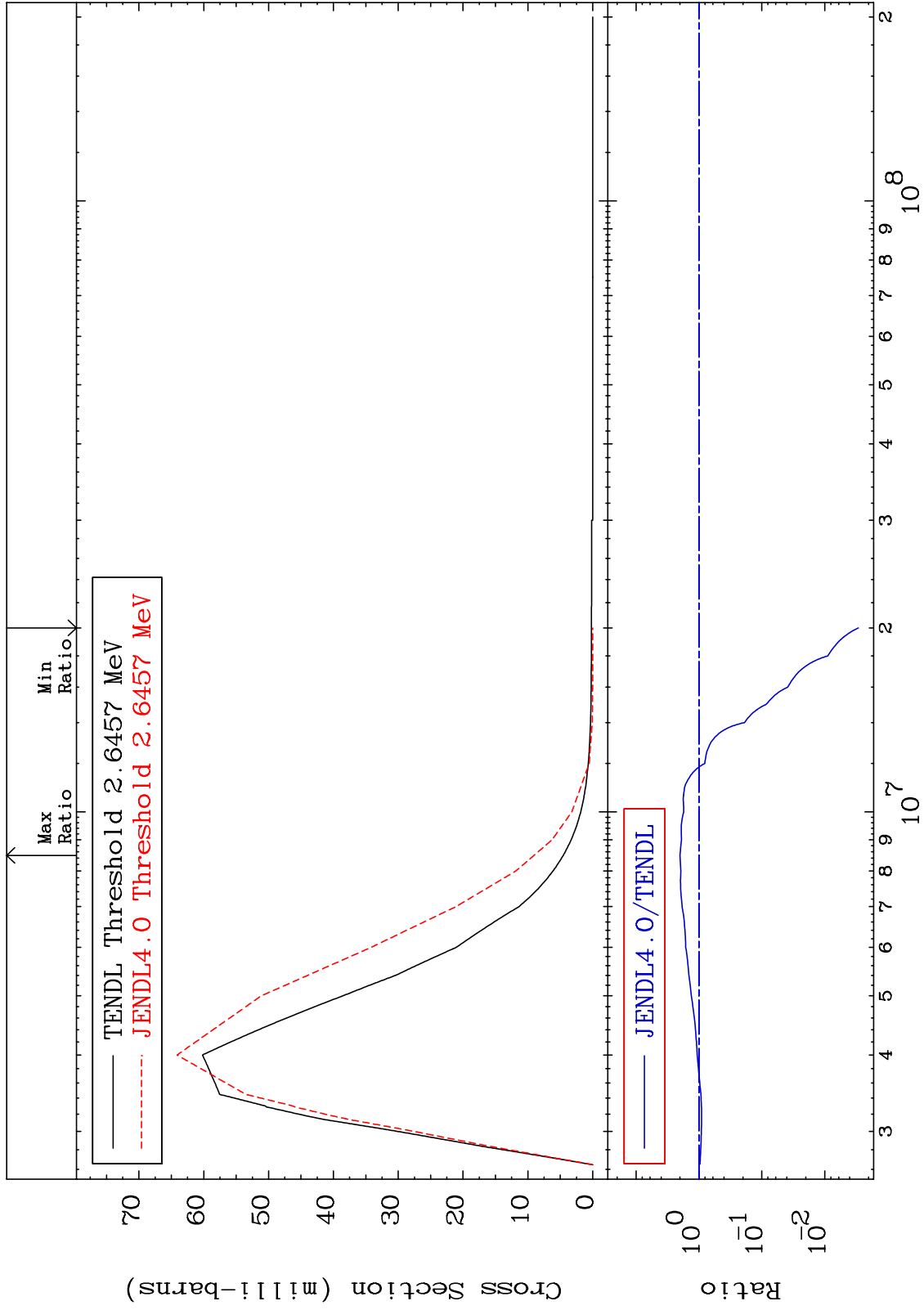
26-Fe-58



MAT 2637

MT= 56 (n,n') Level
Cross Section

26-Fe-58
-99.71 To 100.0 %



12

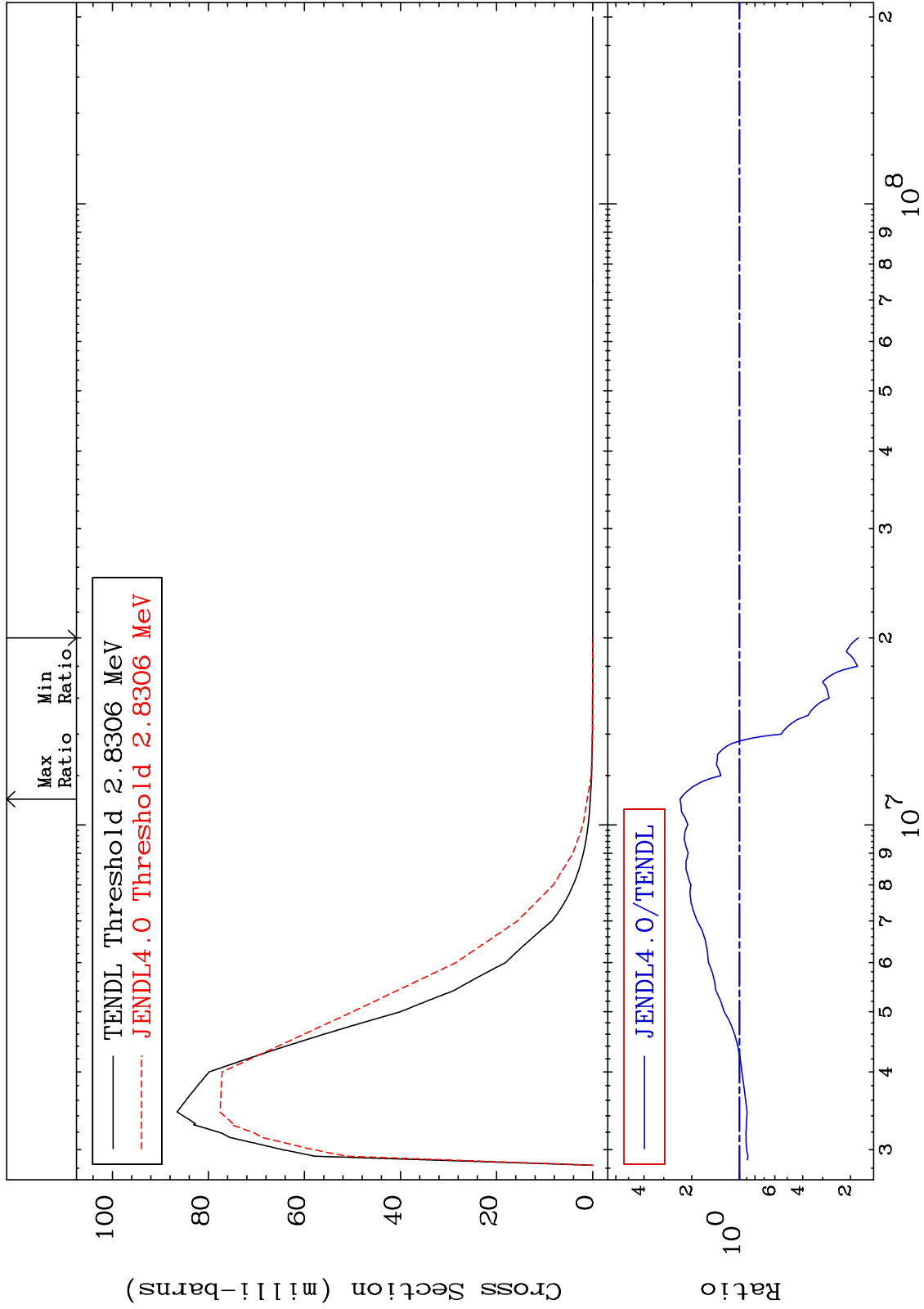
Incident Energy (eV)

26-Fe-58

MAT 2637

MT= 57 (n,n') Level
Cross Section

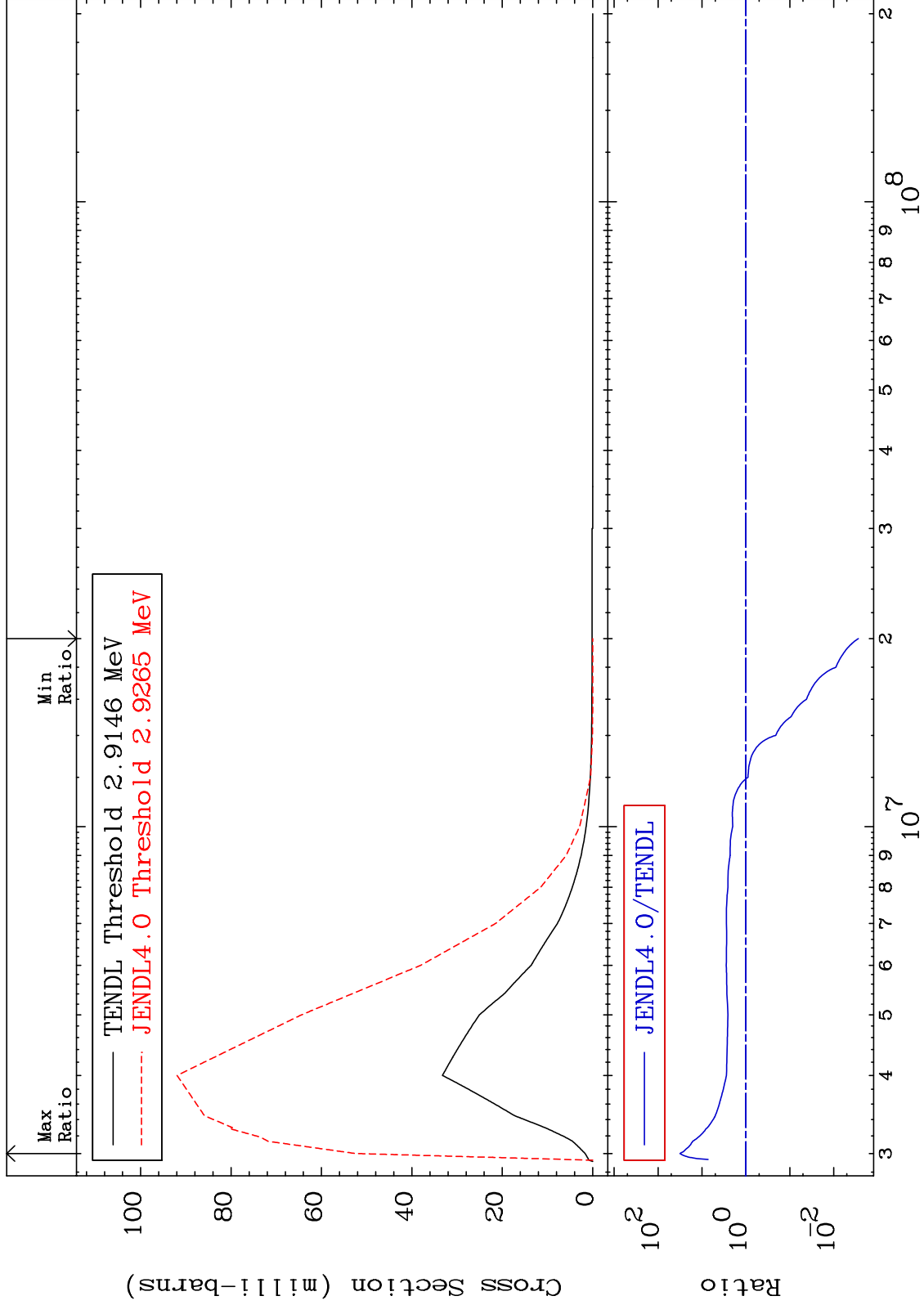
26-Fe-58
-82.16 To 136.9 %



MAT 2637

MT= 58 (n,n') Level
Cross Section

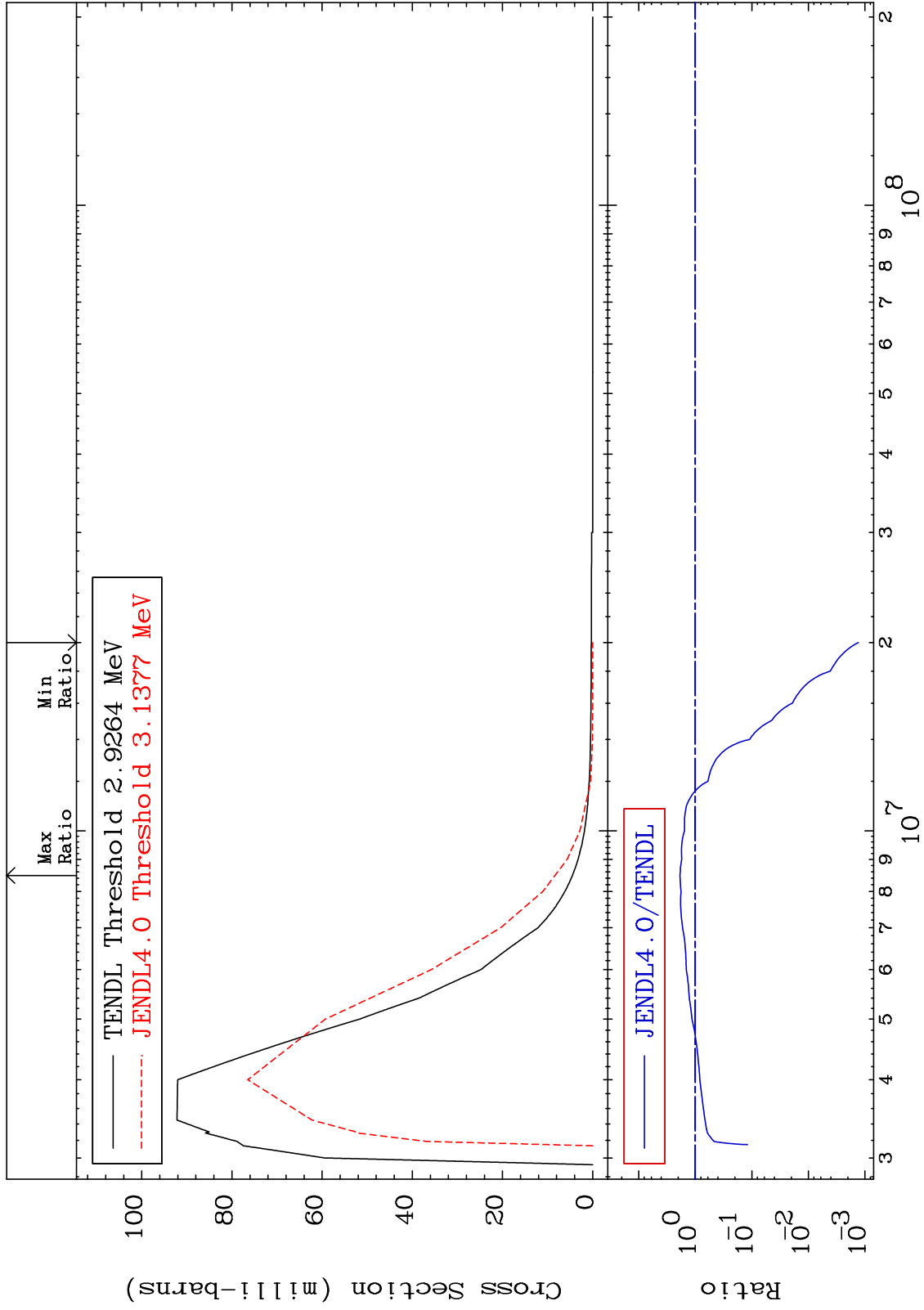
26-Fe-58
-99.73 To 3060. %



MAT 2637

MT= 59 (n,n') Level
Cross Section

26-Fe-58
-99.87 To 86.54 %



15

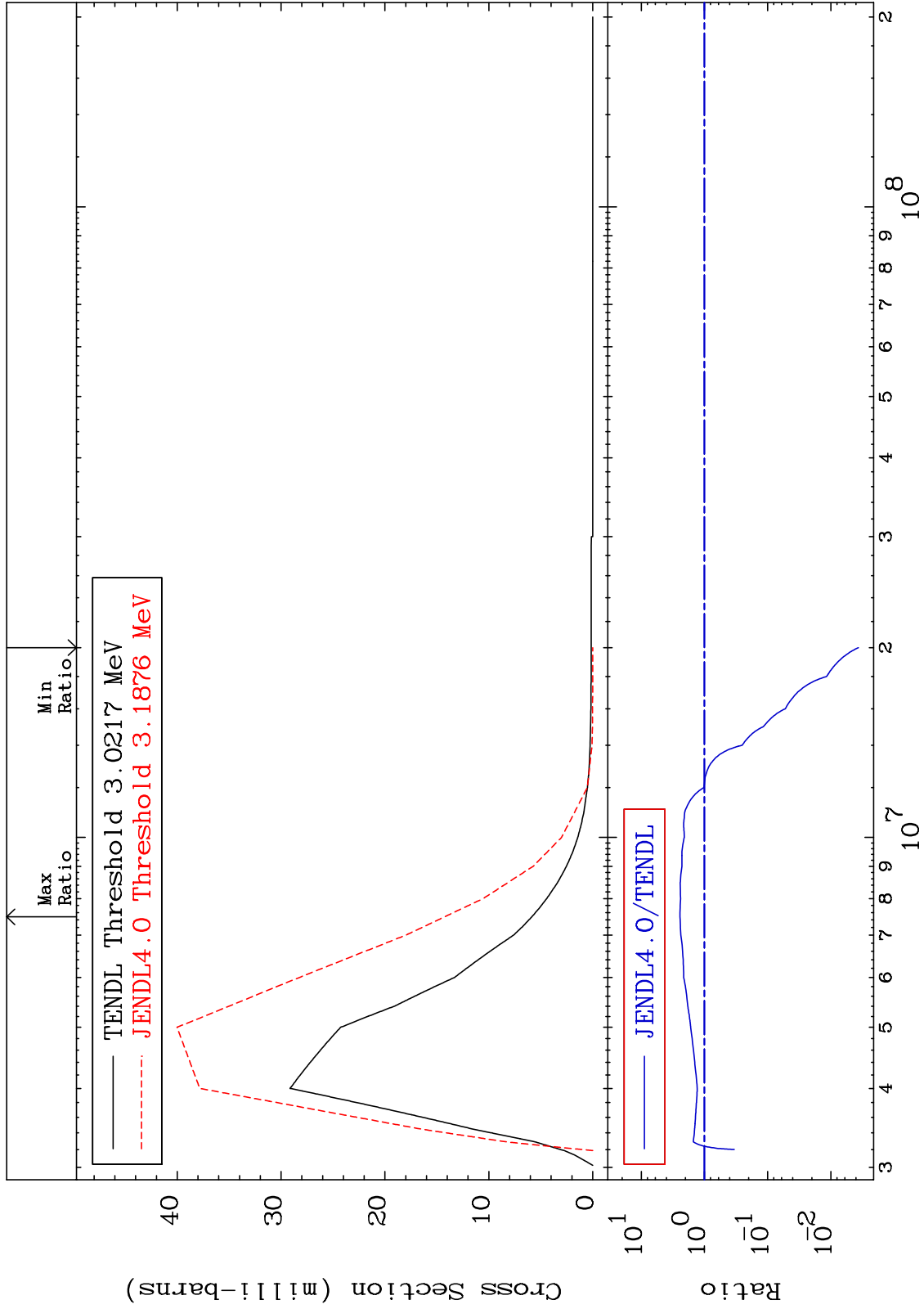
Incident Energy (eV)

26-Fe-58

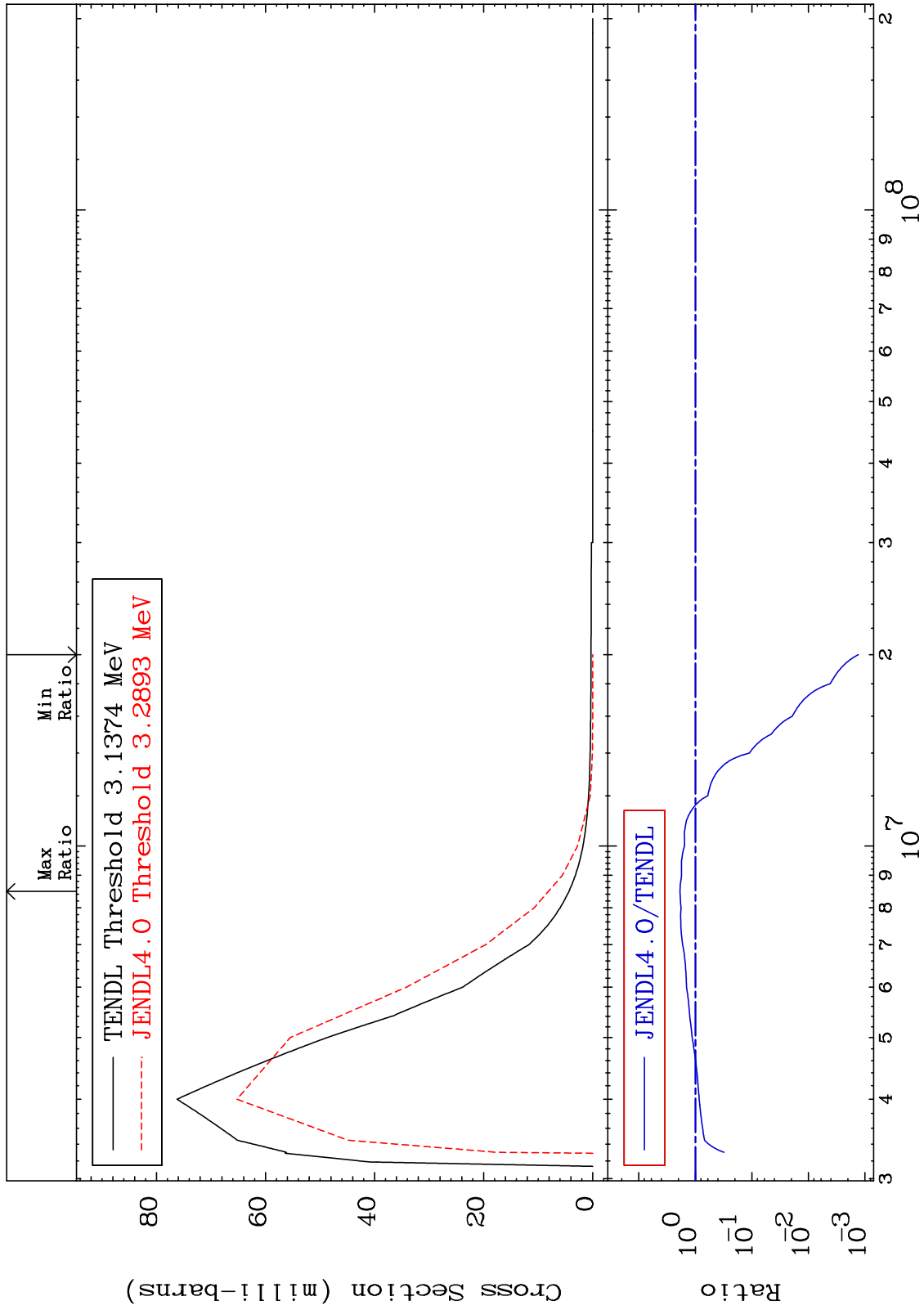
MAT 2637

MT= 60 (n,n') Level
Cross Section

26-Fe-58
-99.63 To 143.4 %



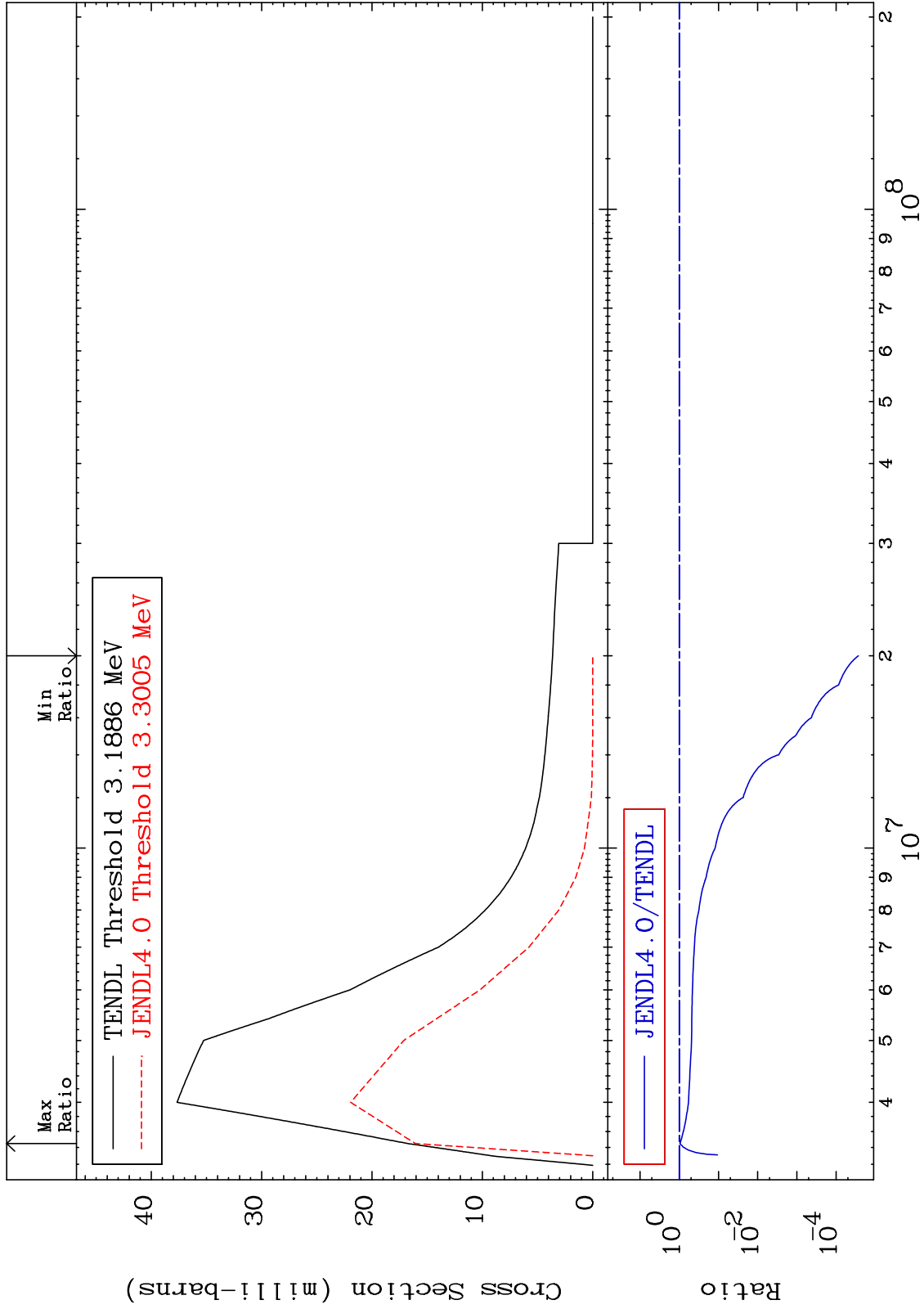
MAT 2637 MT= 61 (n,n') Level 26-Fe-58
 Cross Section -99.87 To 86.70 %



MAT 2637

MT= 62 (n,n') Level
Cross Section

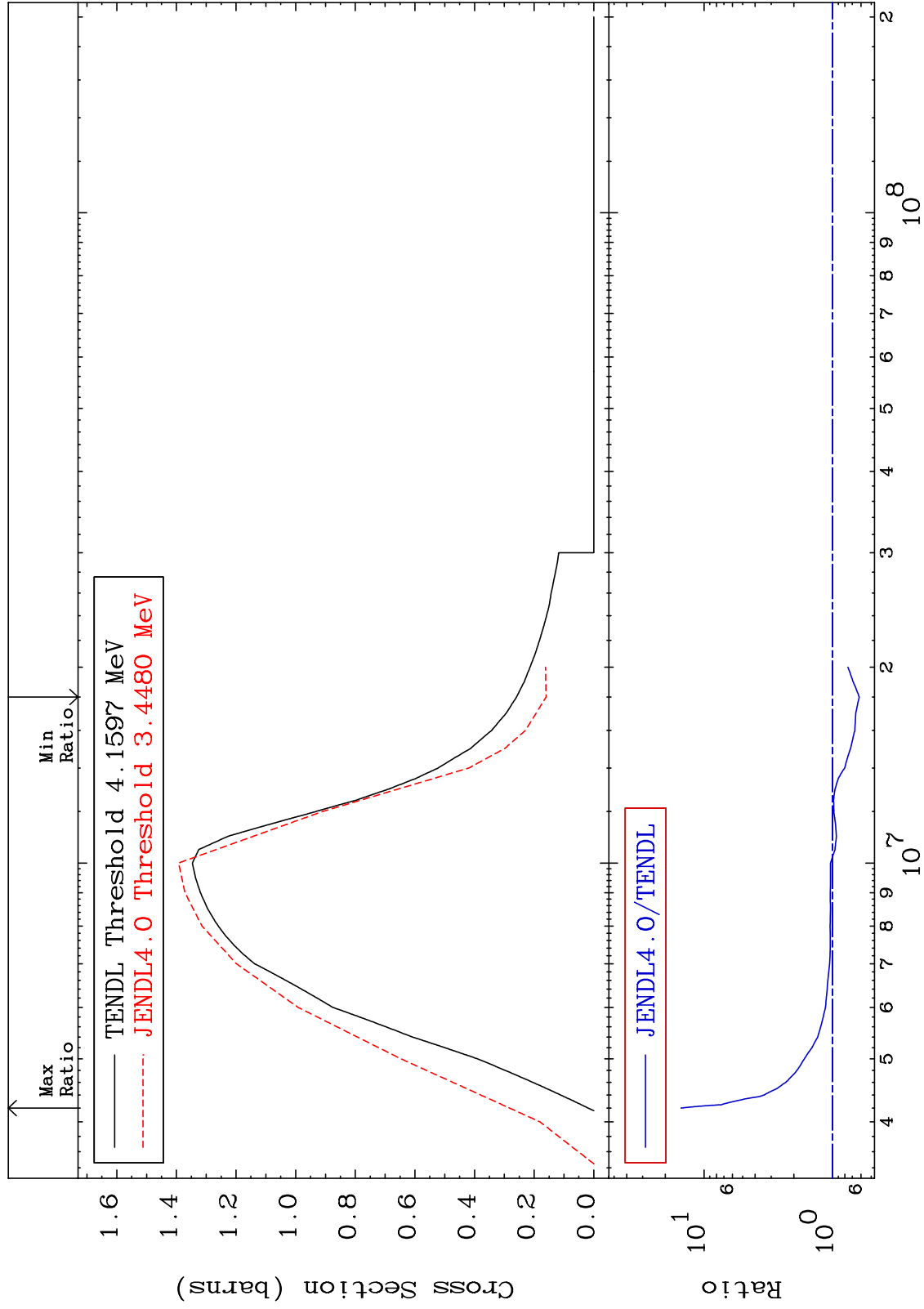
26-Fe-58
-100.0 To -4.100%



MAT 2637

(n, n') Continuum
Cross Section

²⁶Fe-58
-38.19 To 1408. %

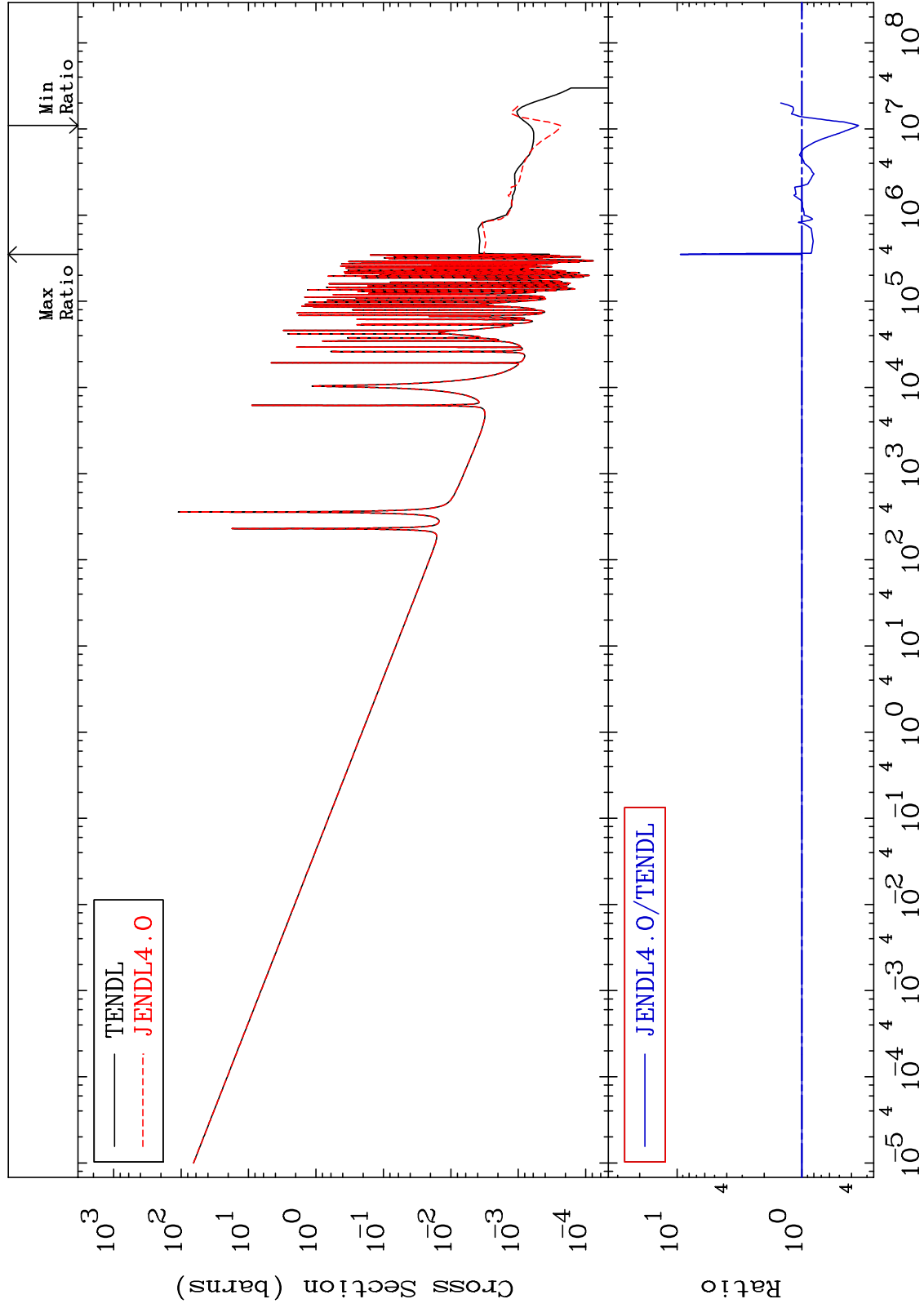


MAT 2637

(n, γ)
Cross Section

26-Fe-58

-64.89 To 840.2 %



— TENDL
- - - JENDL4.0

— JENDL4.0/TENDL

26-Fe-58

Incident Energy (eV)

20

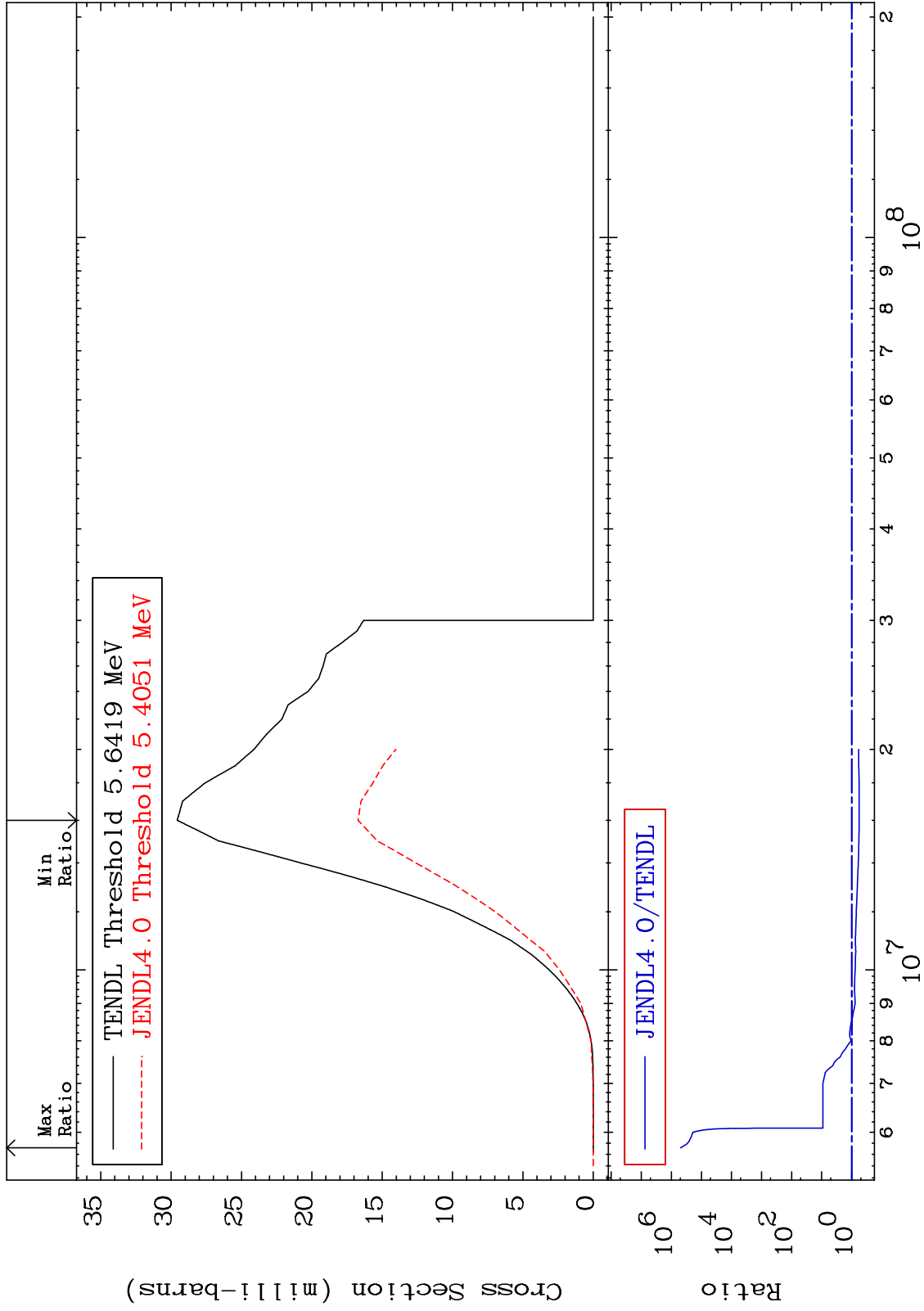
MAT 2637

(n,p)

²⁶Fe-58

Cross Section

-43.46 To 9999. %



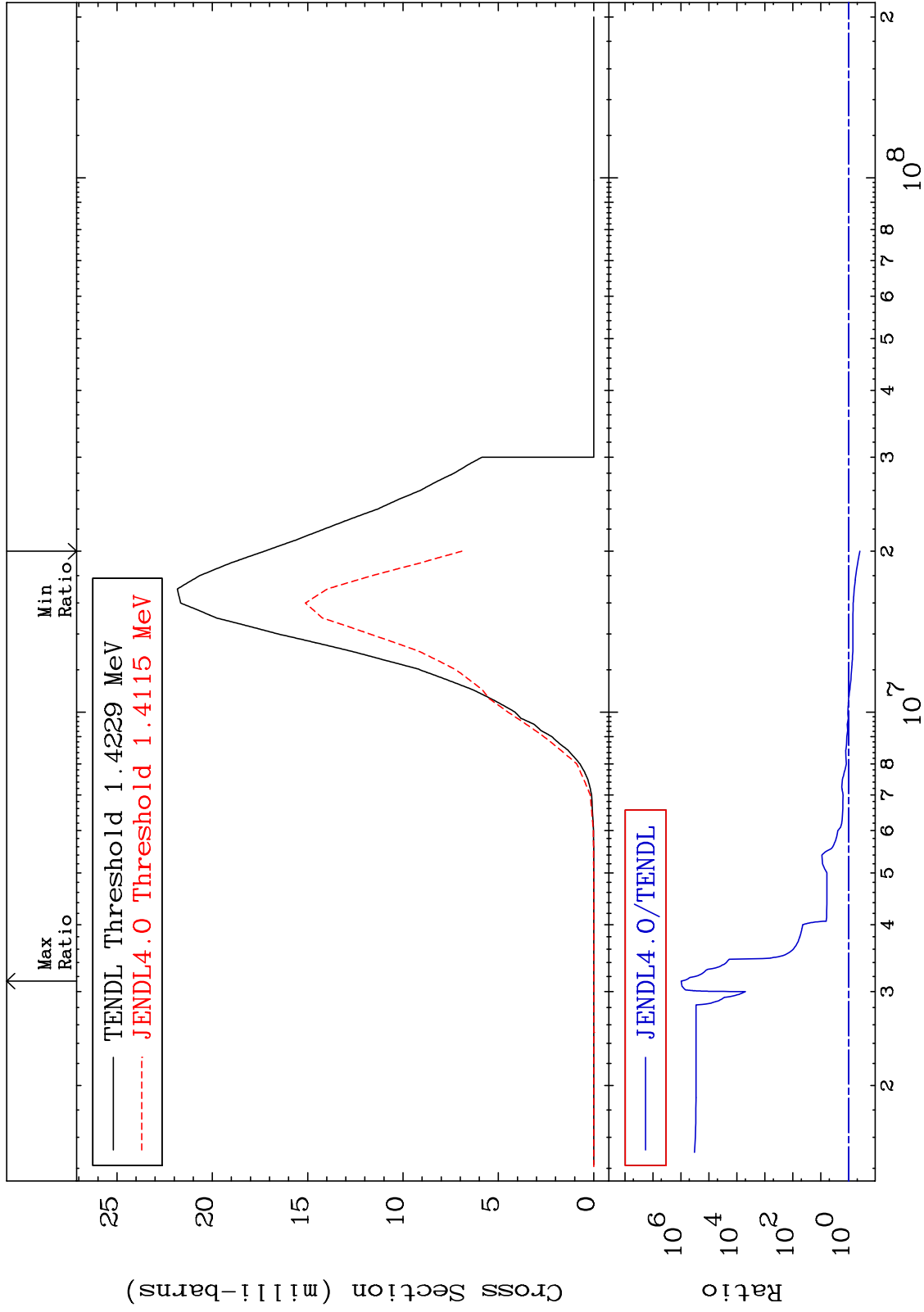
MAT 2637

(n, α)

²⁶Fe-58

Cross Section

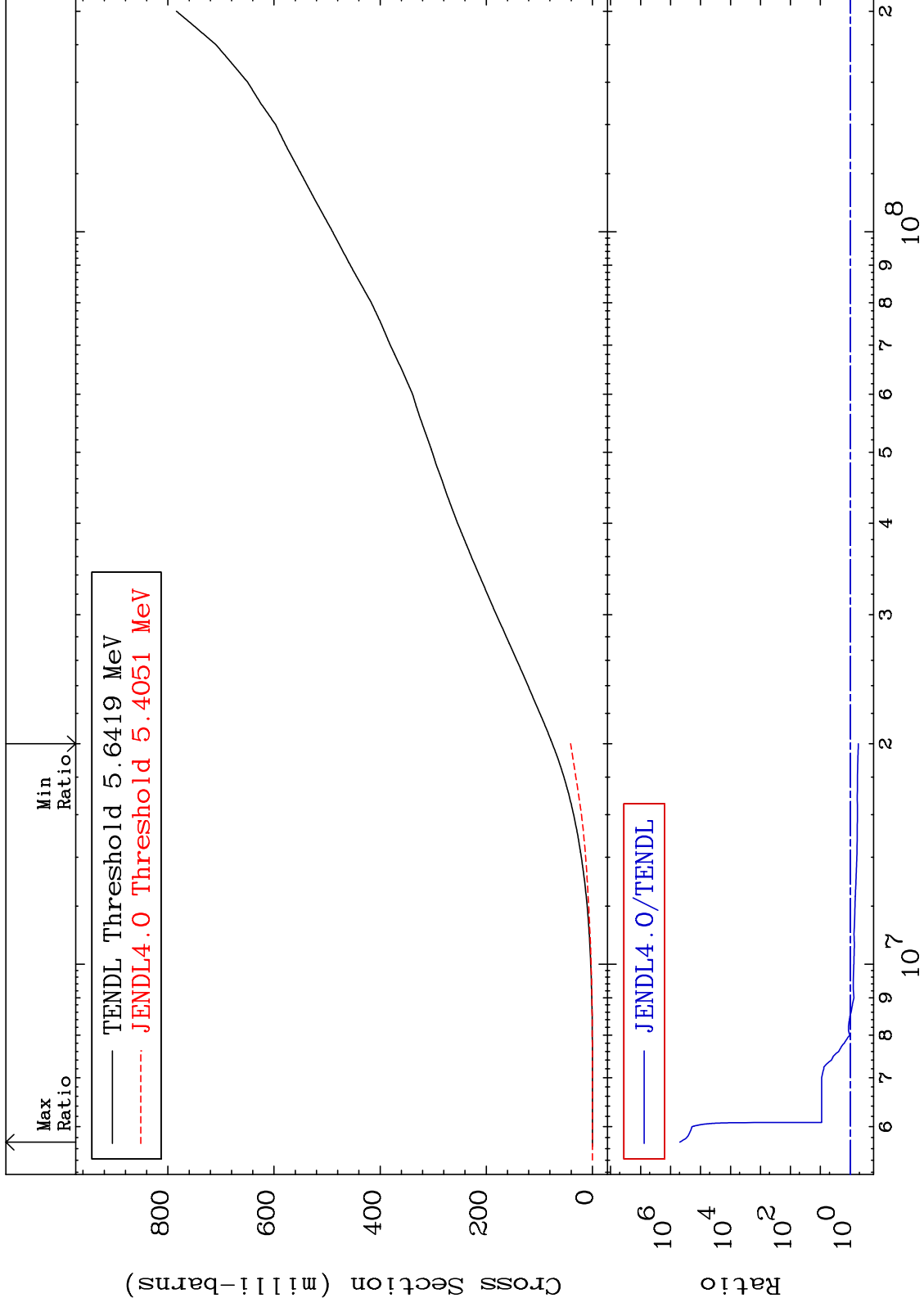
-59.92 To 9999. %



MAT 2637

Hydrogen Production
Cross Section

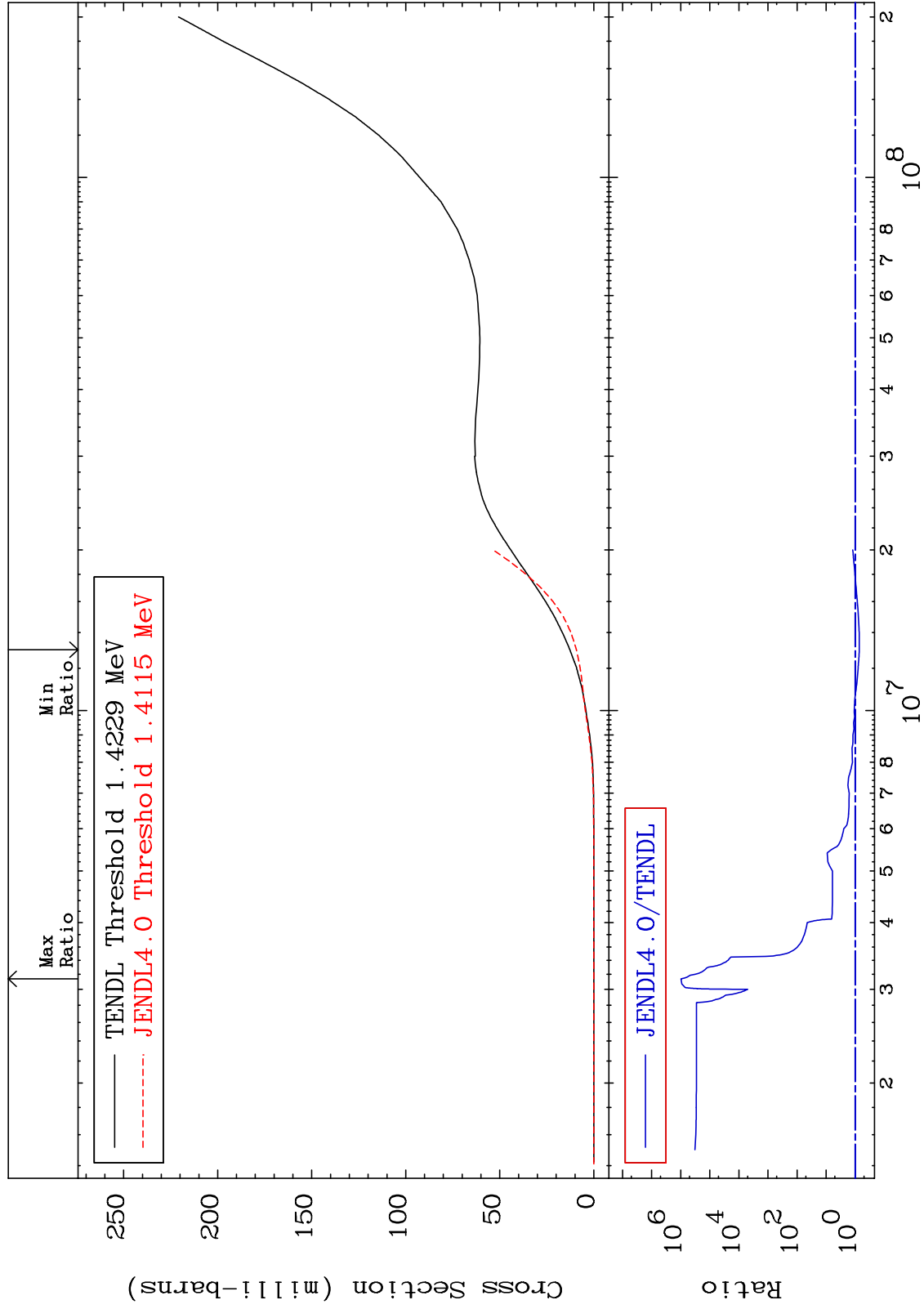
26-Fe-58
-45.79 To 9999. %



MAT 2637

He-4 Production
Cross Section

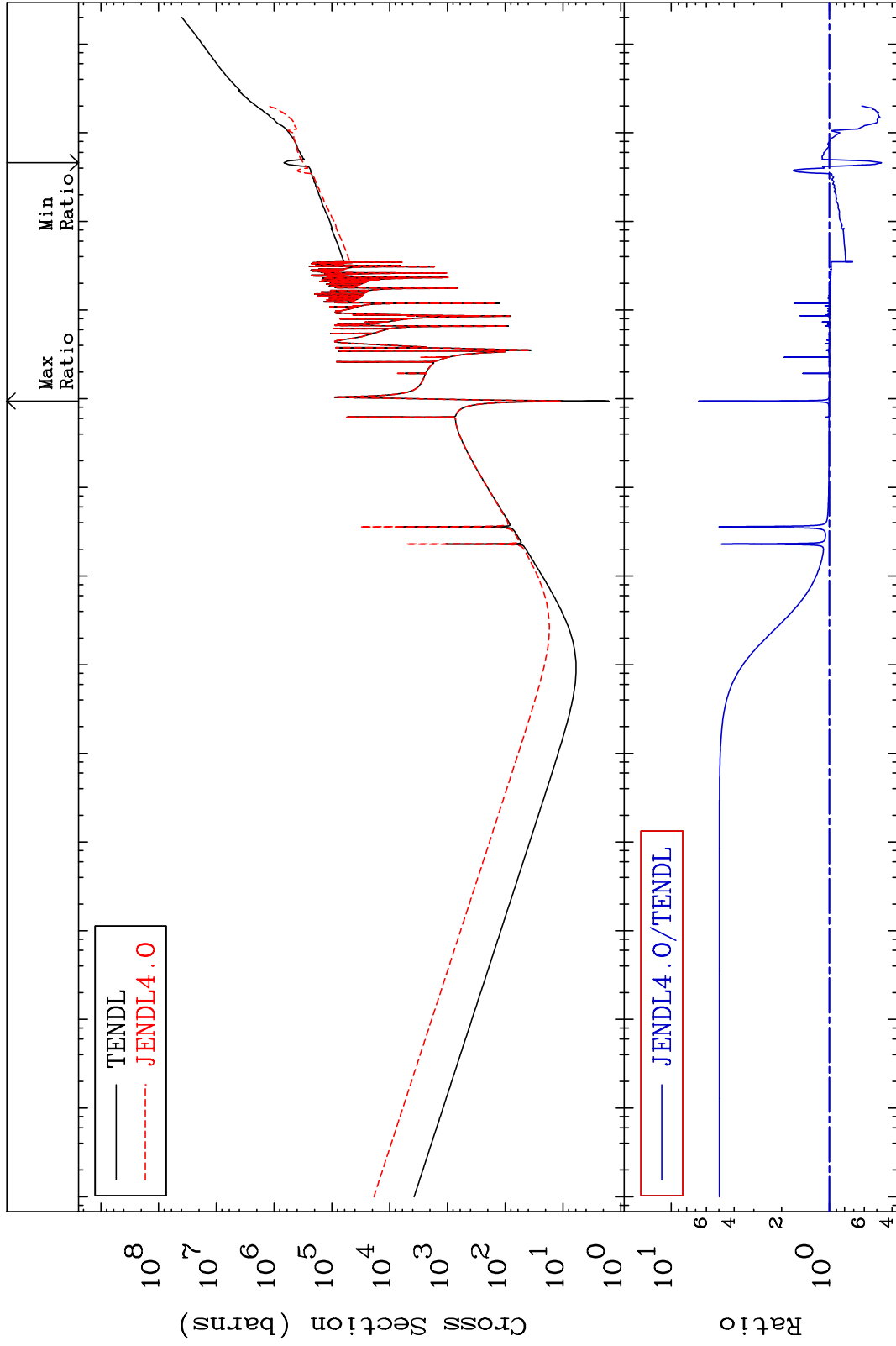
26-Fe-58
-27.59 To 9999. %



MAT 2637

Kerma total (eV-barns)
Cross Section

26-Fe-58
-53.34 To 573.4 %



25

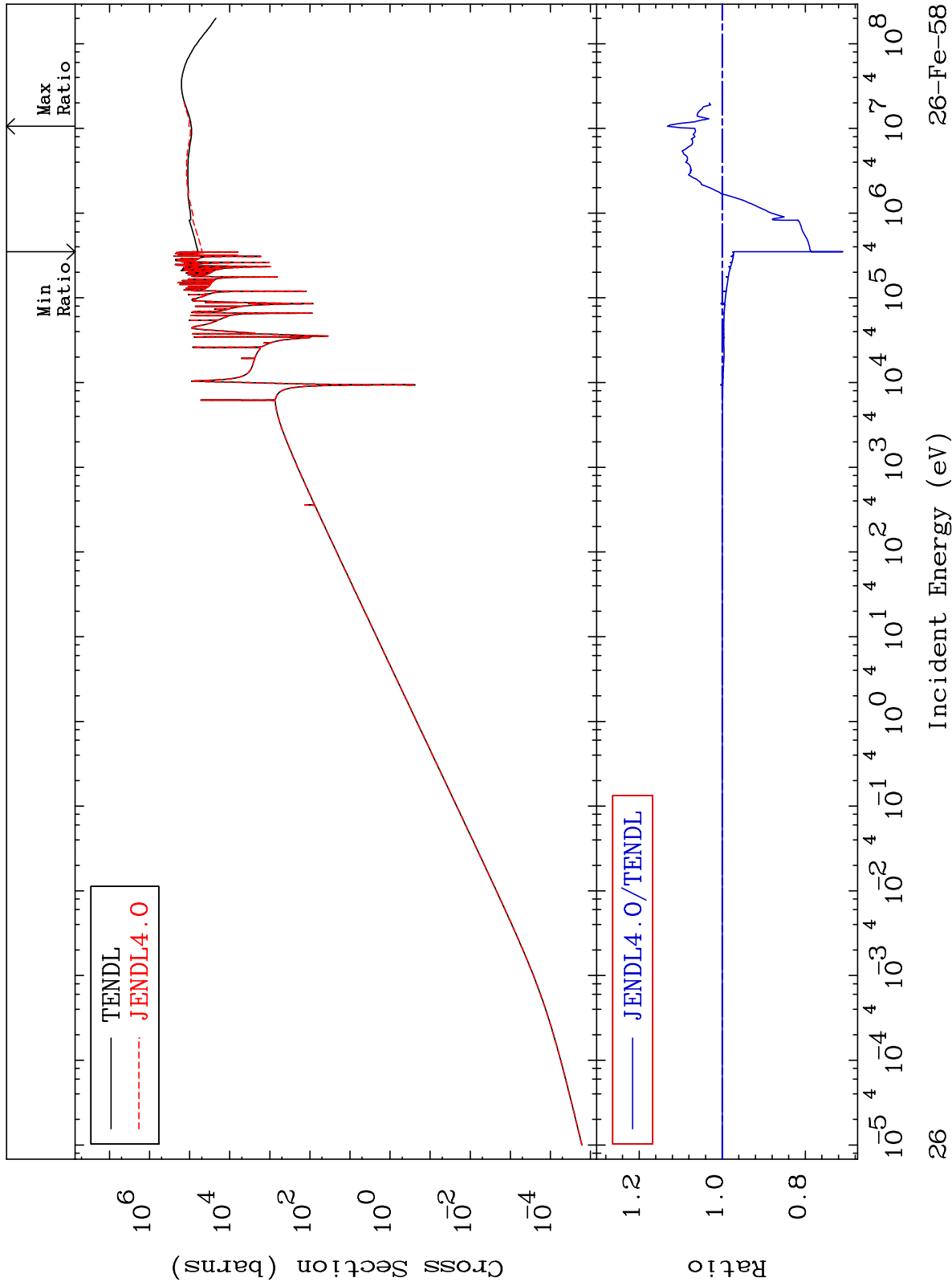
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma elastic
Cross Section

26-Fe-58
-28.99 To 13.16 %



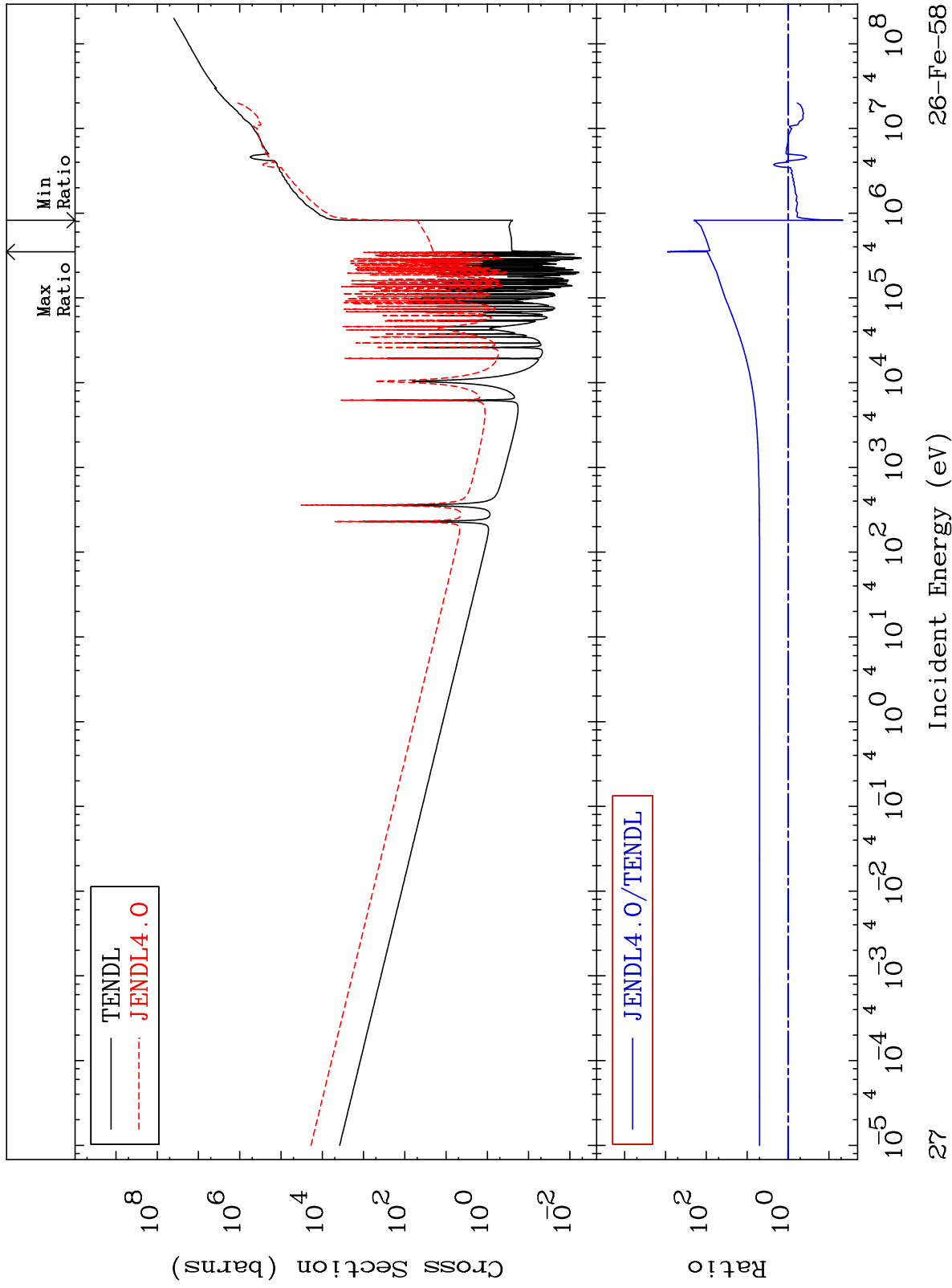
26

26-Fe-58

MAT 2637

Kerma non-elastic (all but mt2)
Cross Section

26-Fe-58
-95.41 To 9999. %



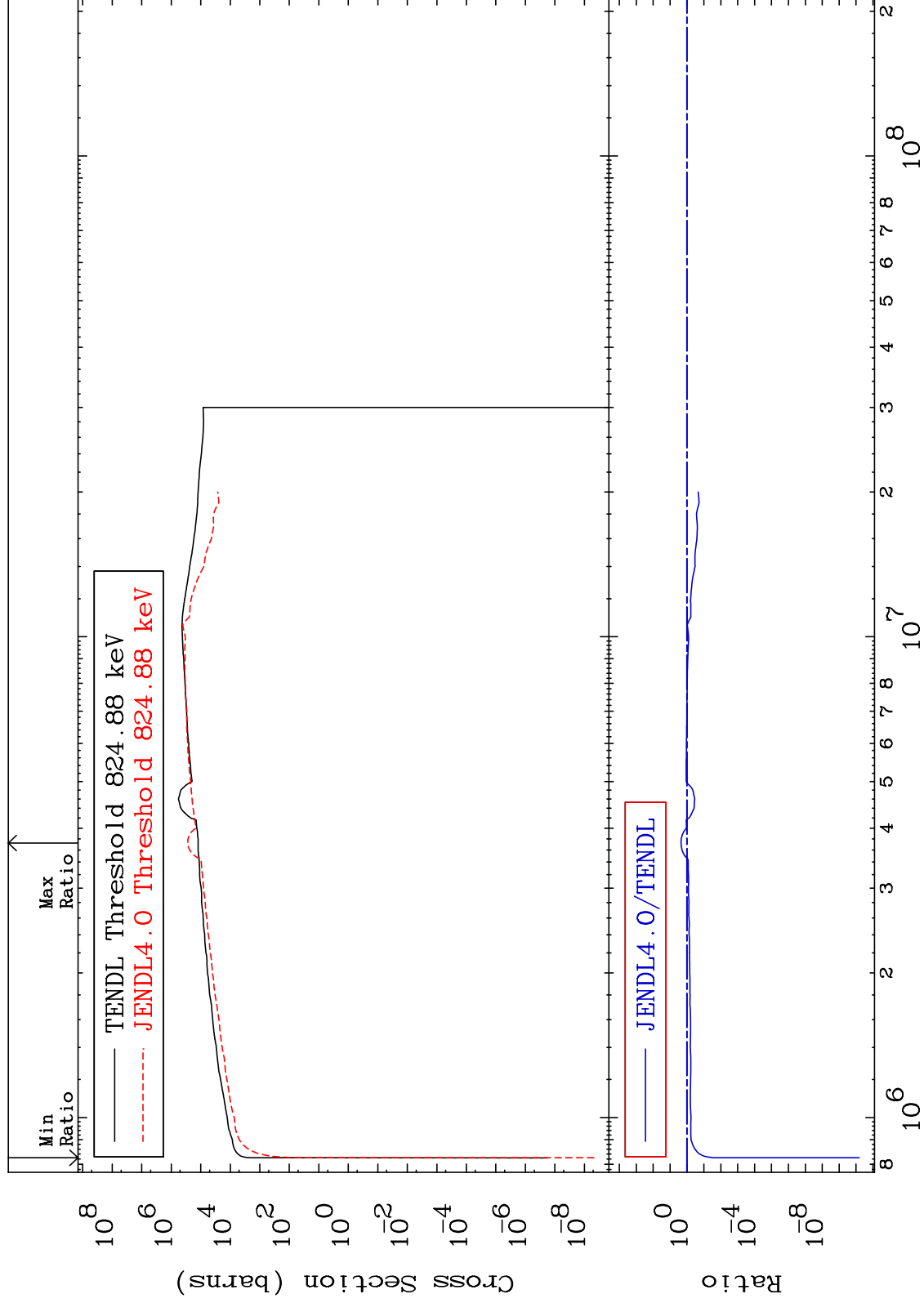
27

26-Fe-58

MAT 2637

Kerma inelastic (mt51-91)
Cross Section

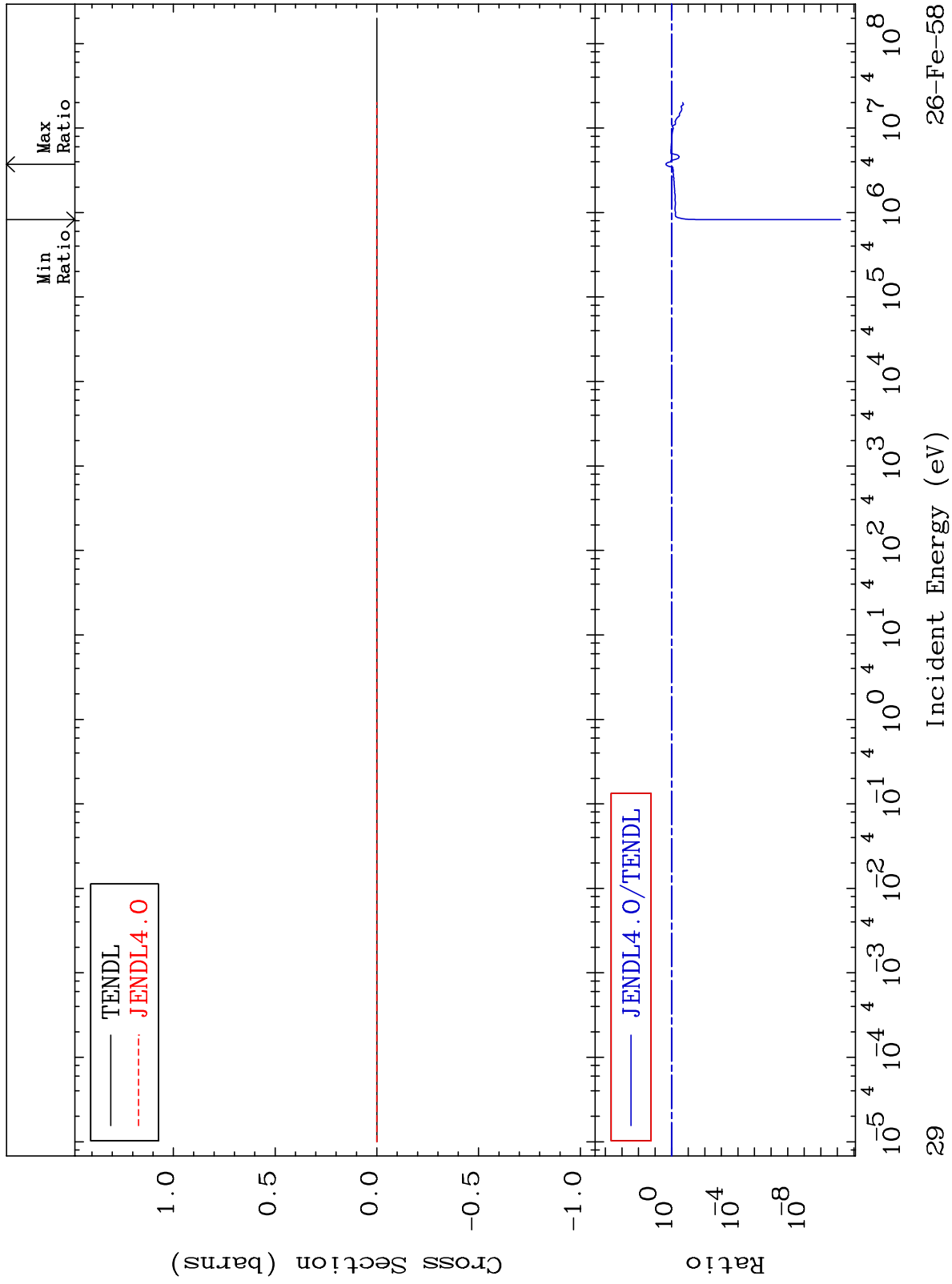
26-Fe-58
-100.0 To 123.3 %



MAT 2637

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

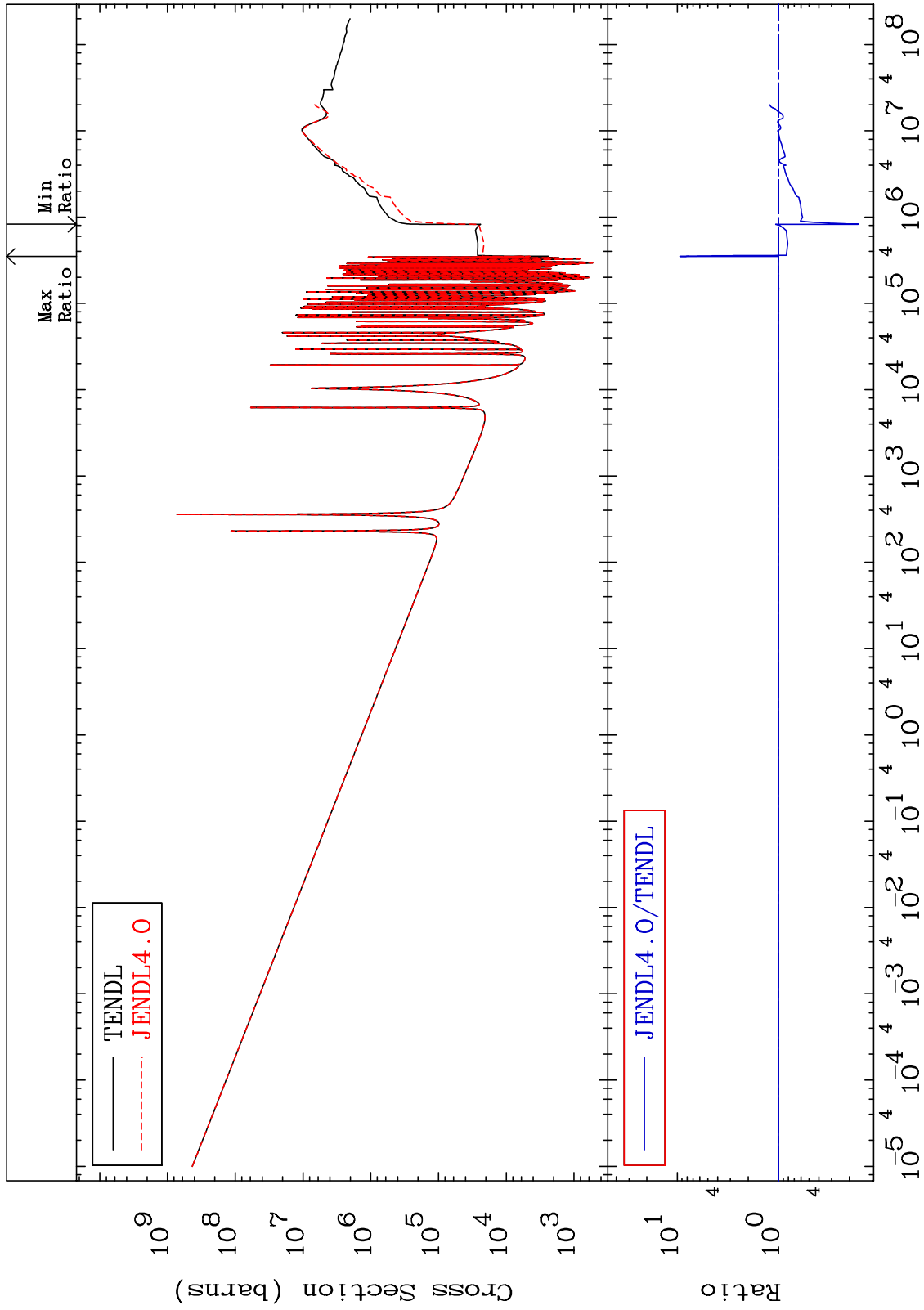
26-Fe-58
-100.0 To 123.3 %



MAT 2637

Total photon (eV-barns)
Cross Section

26-Fe-58
-83.70 To 841.0 %



31

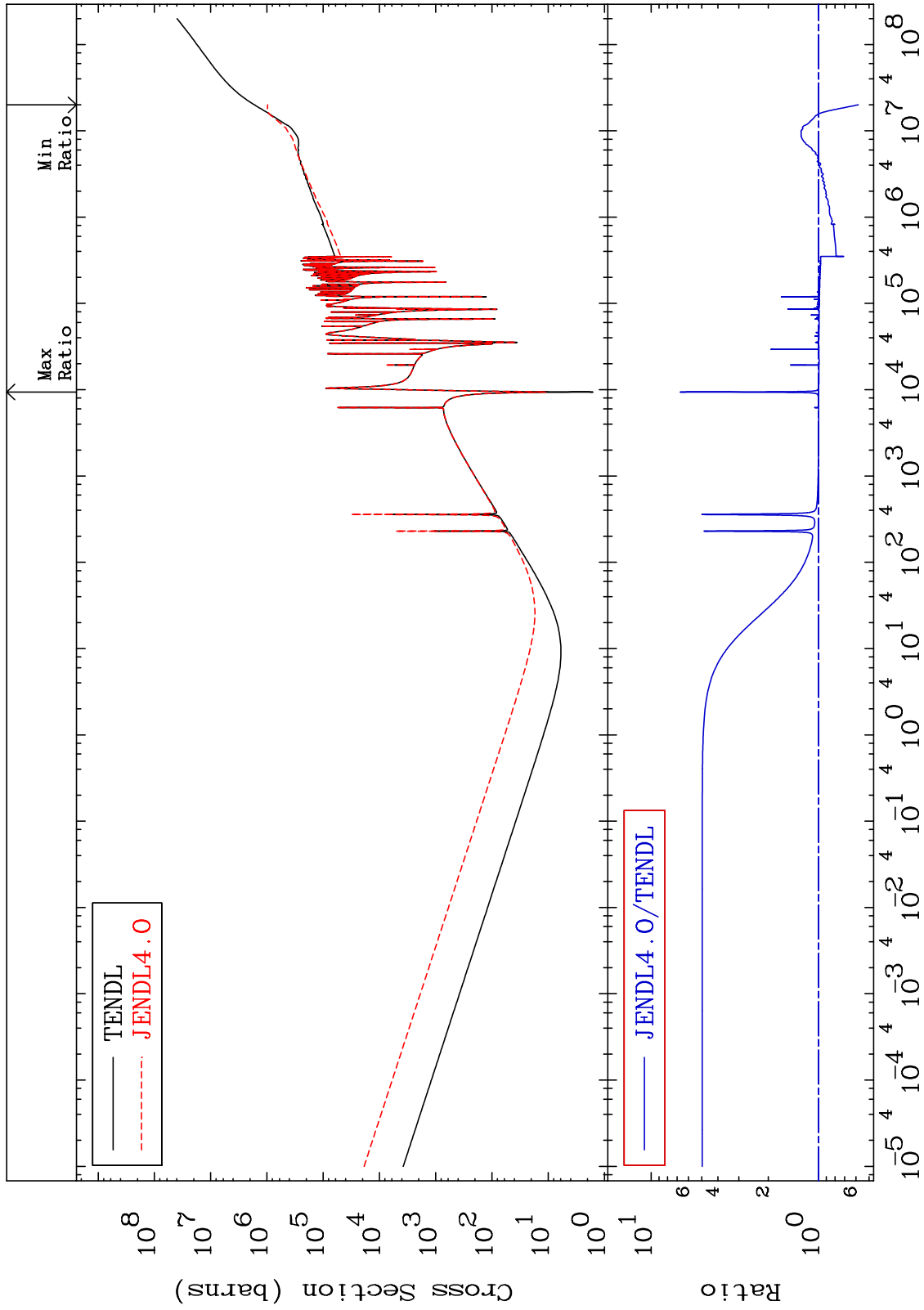
Incident Energy (eV)

26-Fe-58

MAT 2637

Total kinematic kerma (high limit)
Cross Section

26-Fe-58
-42.07 To 573.4 %



32

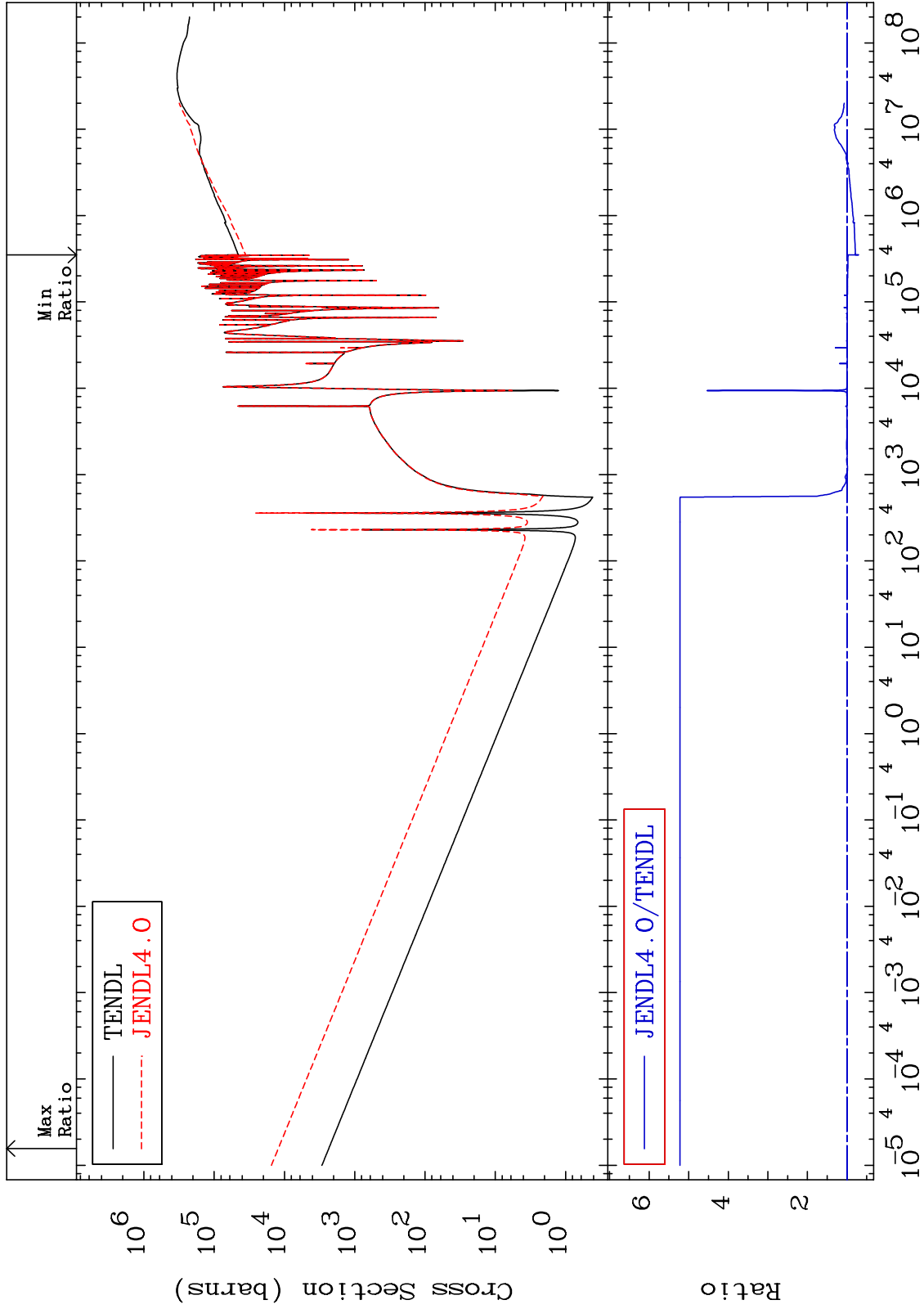
Incident Energy (eV)

26-Fe-58

MAT 2637

Dpa total (eV-barns)
Cross Section

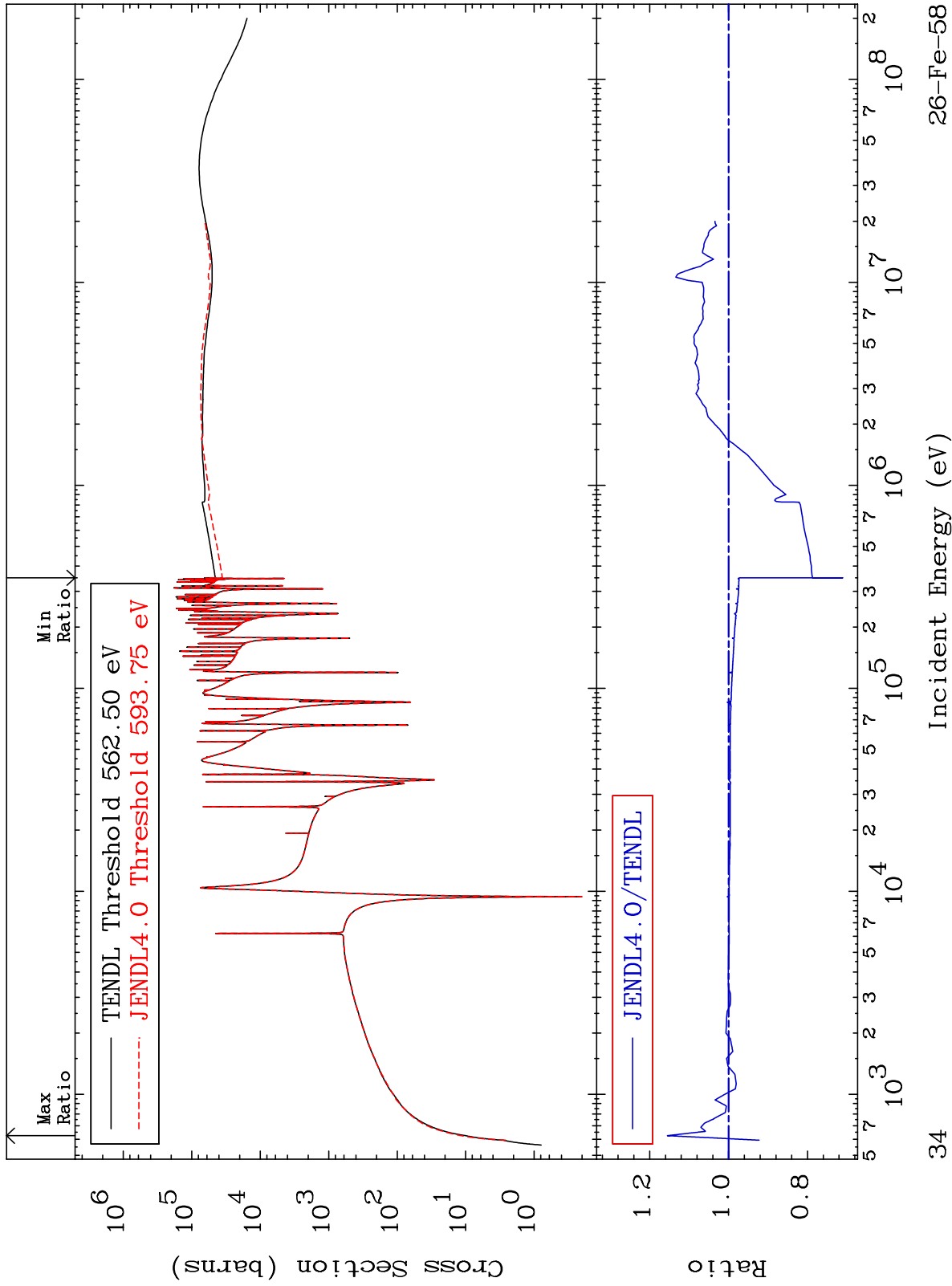
26-Fe-58
-28.91 To 421.9 %



MAT 2637

Dpa elastic (mt2)
Cross Section

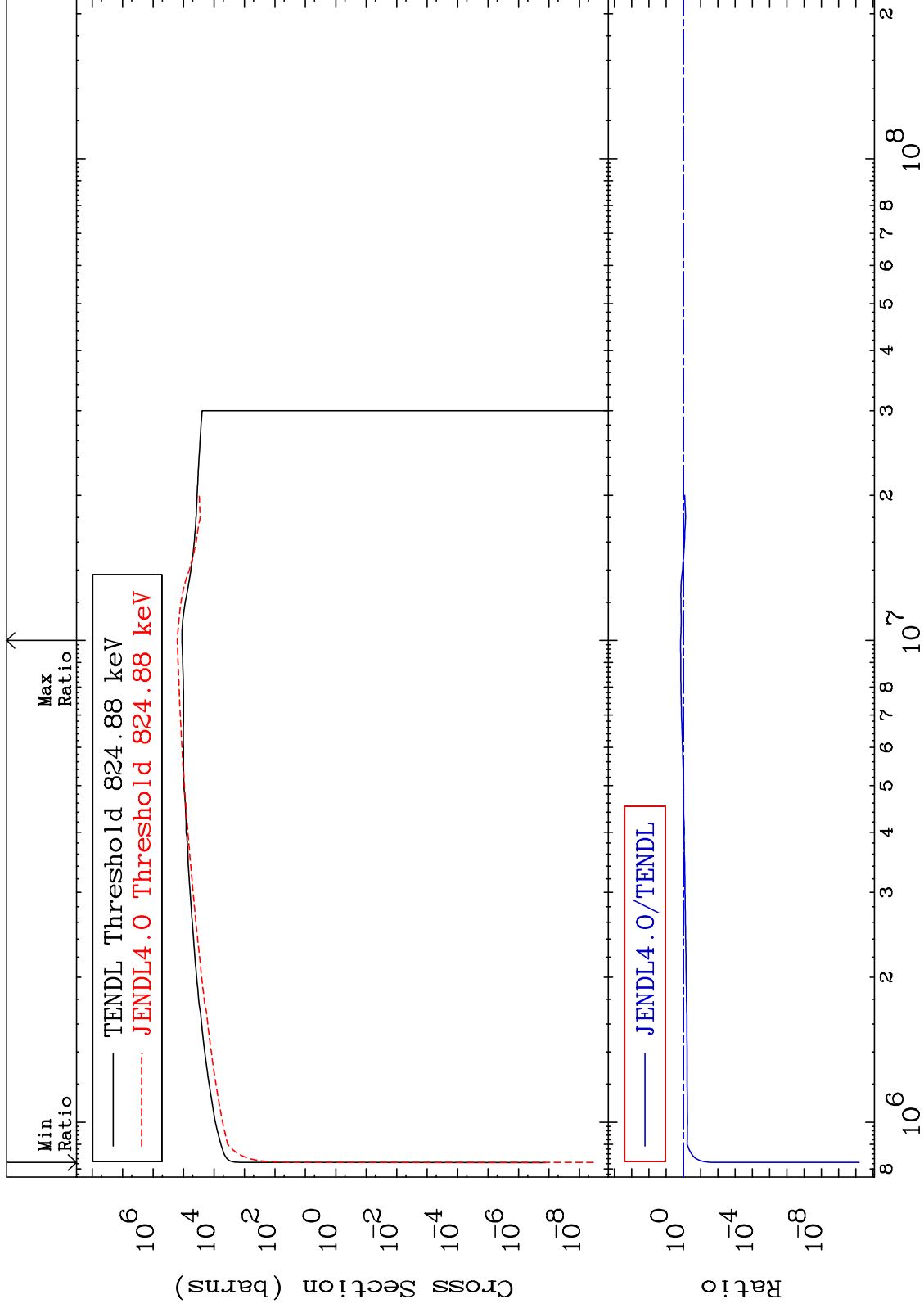
26-Fe-58
-28.91 To 15.47 %



MAT 2637

Dpa inelastic (mt51-91)
Cross Section

26-Fe-58
-100.0 To 43.78 %



35

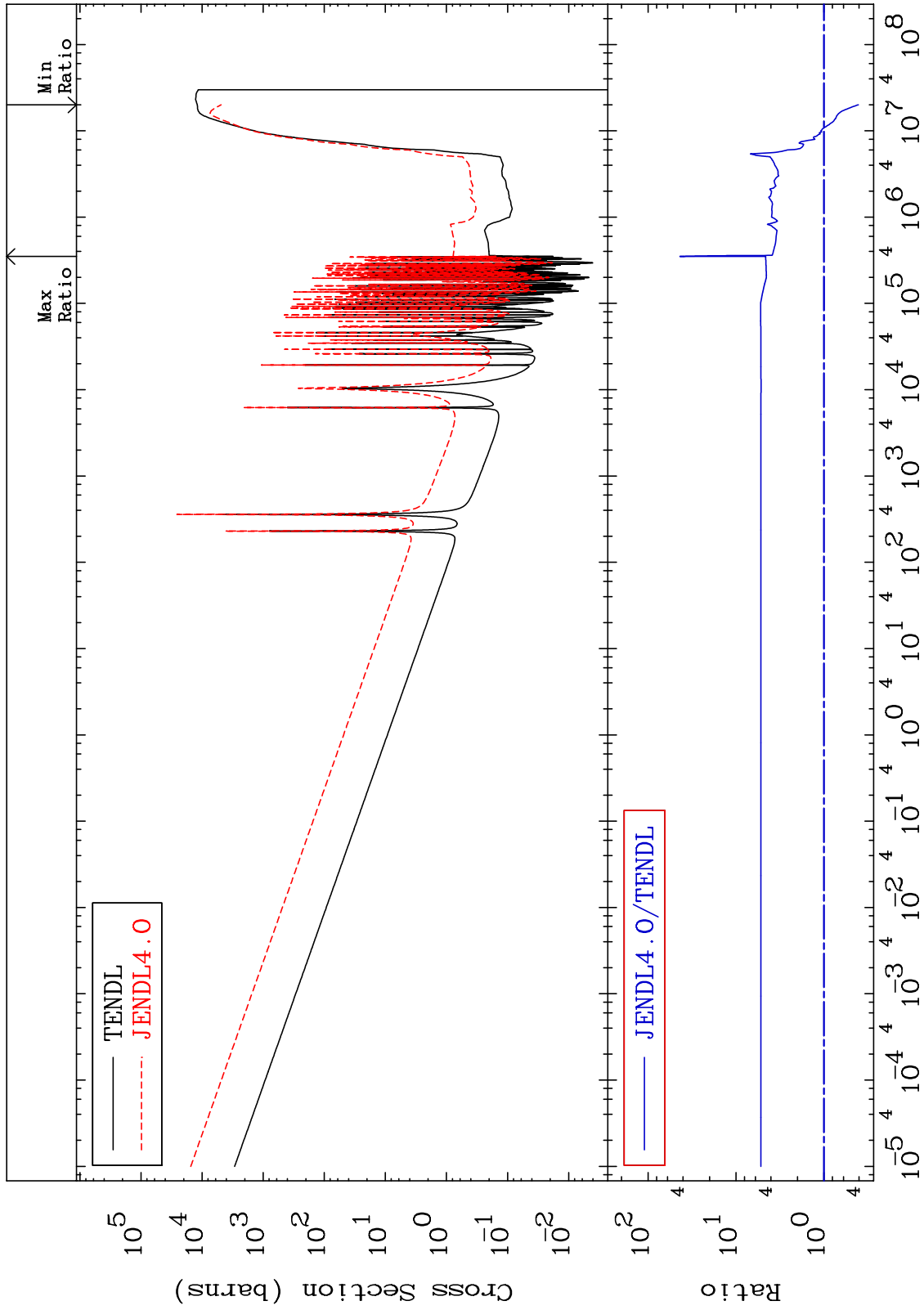
26-Fe-58

26-Fe-58

MAT 2637

Dpa disappearance (mt102 -120)
Cross Section

26-Fe-58
-59.42 To 4228. %



36

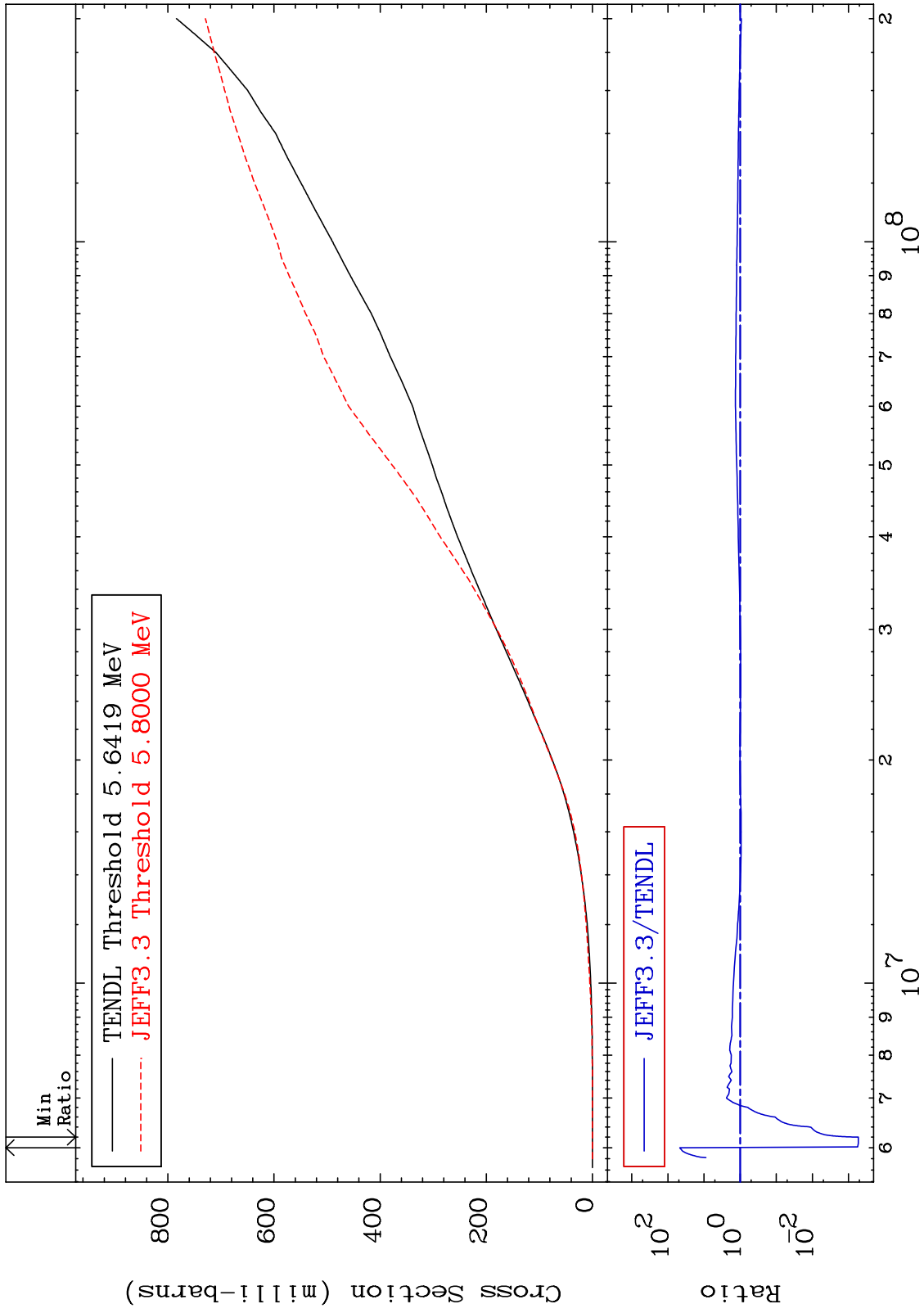
Incident Energy (eV)

26-Fe-58

MAT 2637

Hydrogen Production
Cross Section

26-Fe-58
-99.95 To 4594. %



37

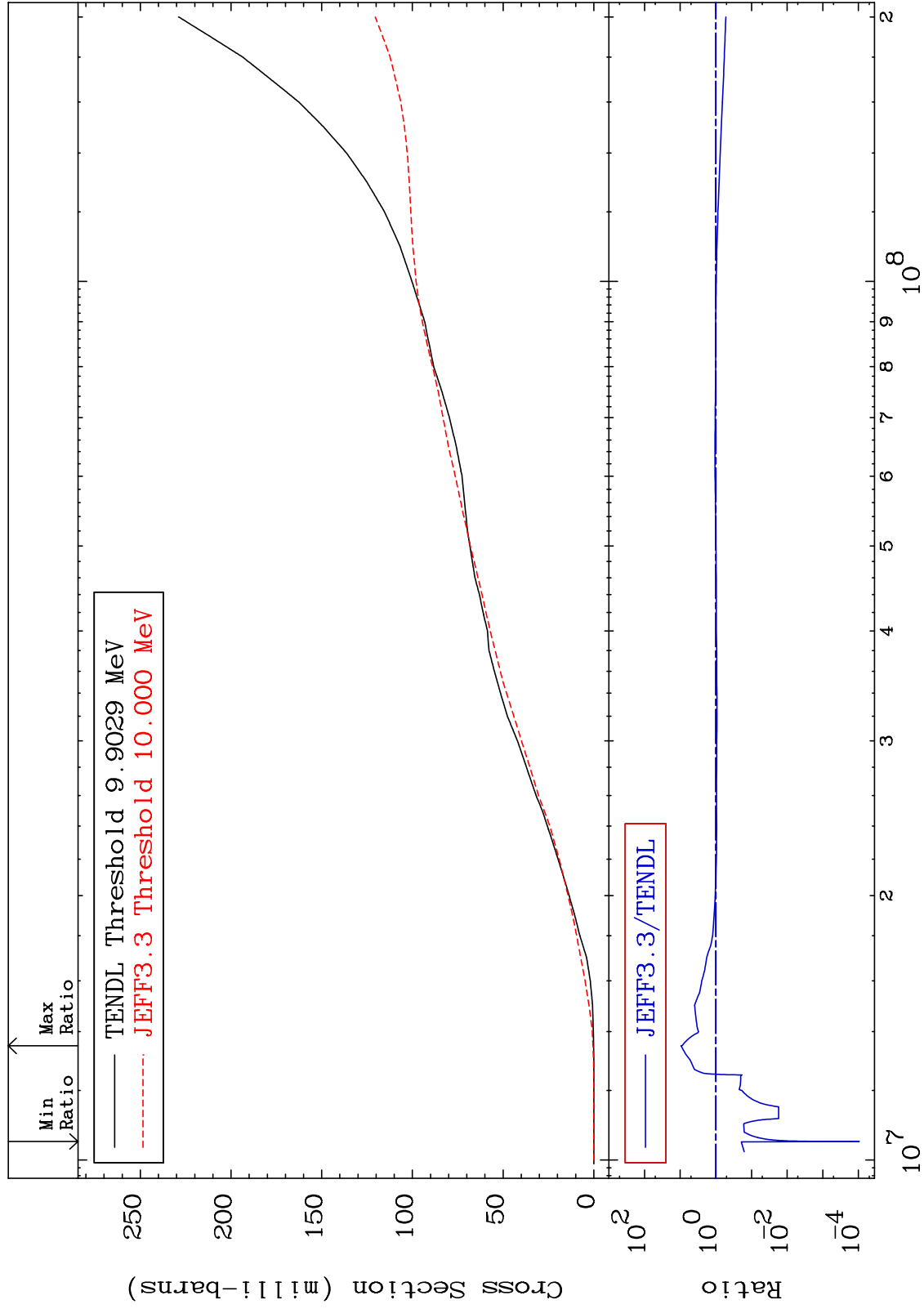
Incident Energy (eV)

26-Fe-58

MAT 2637

Deuterium Production
Cross Section

$^{26}\text{Fe-58}$
-99.99 To 845.4 %



$^{26}\text{Fe-58}$

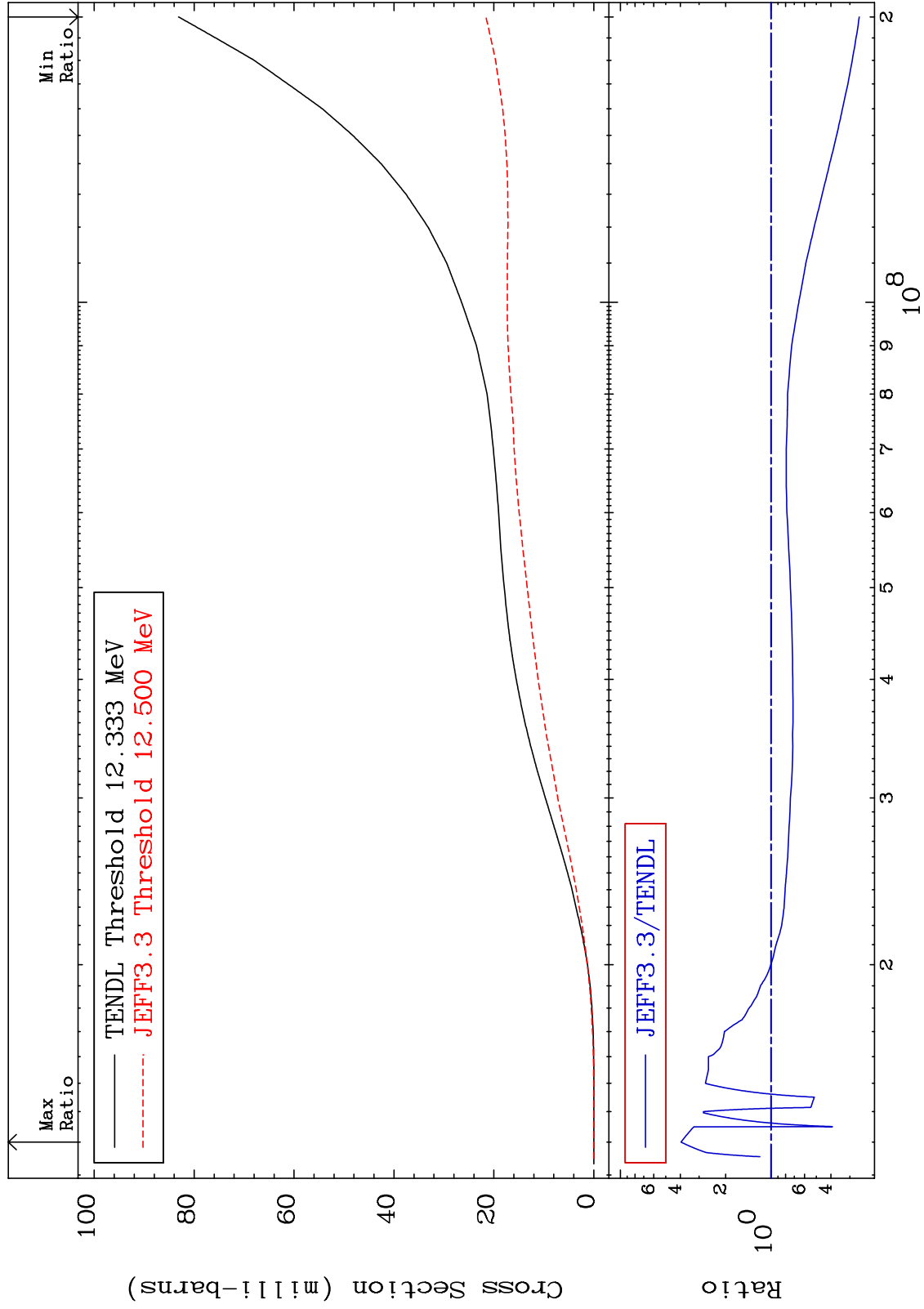
Incident Energy (eV)

38

MAT 2637

Tritium Production
Cross Section

²⁶Fe-58
-74.04 To 295.6 %



39

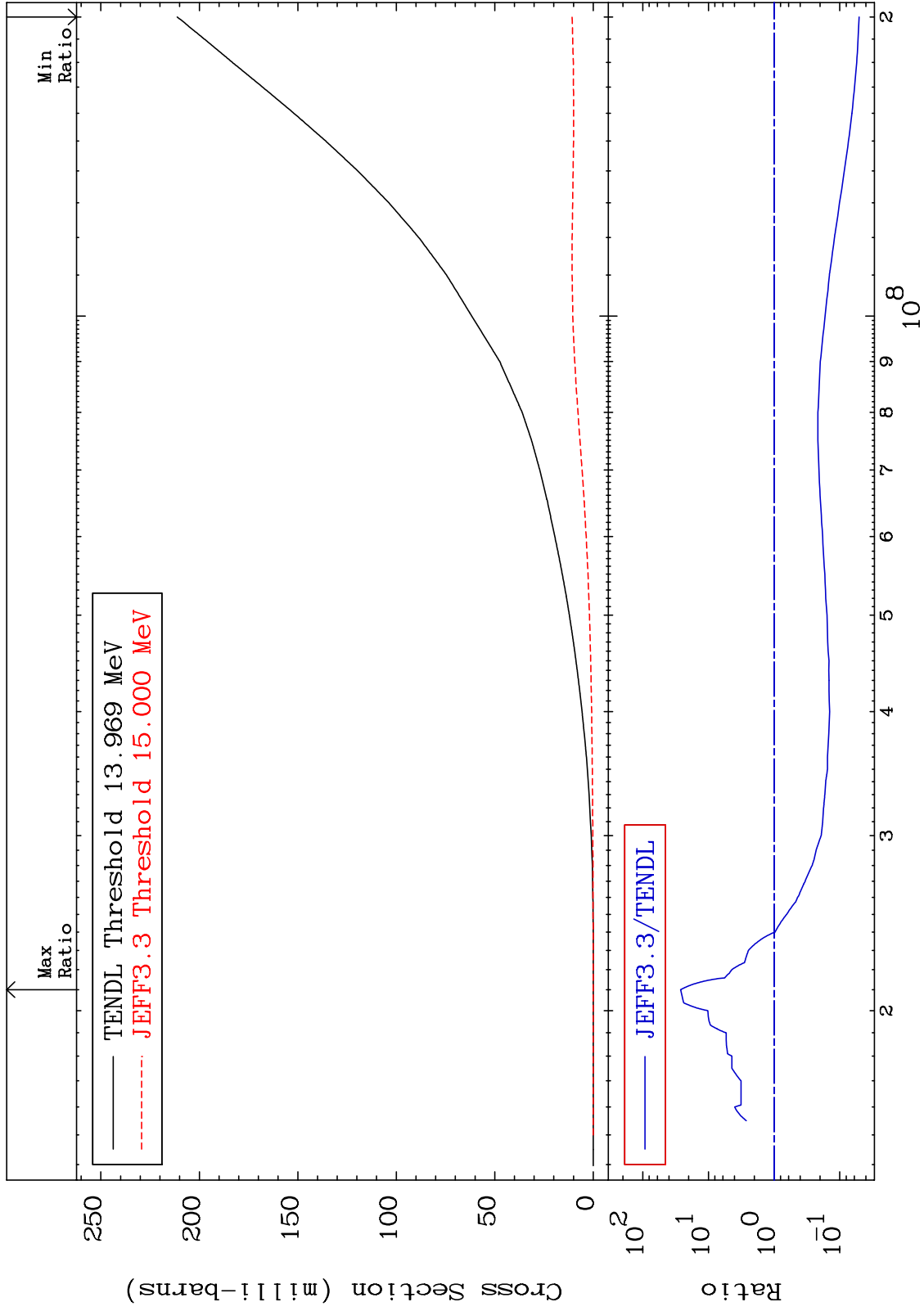
Incident Energy (eV)

²⁶Fe-58

MAT 2637

He-3 Production
Cross Section

26-Fe-58
-94.93 To 2552. %



40

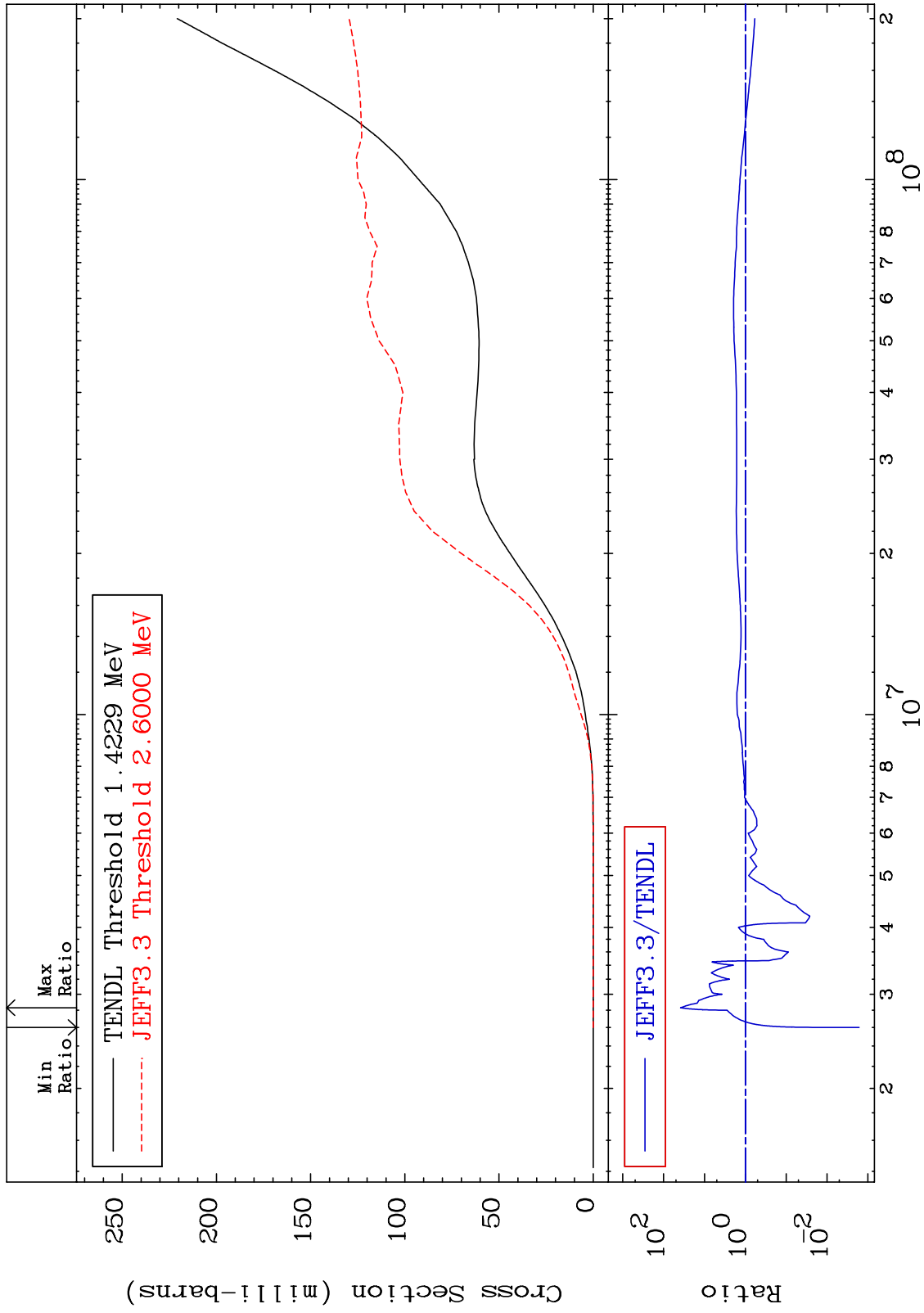
Incident Energy (eV)

26-Fe-58

MAT 2637

He-4 Production
Cross Section

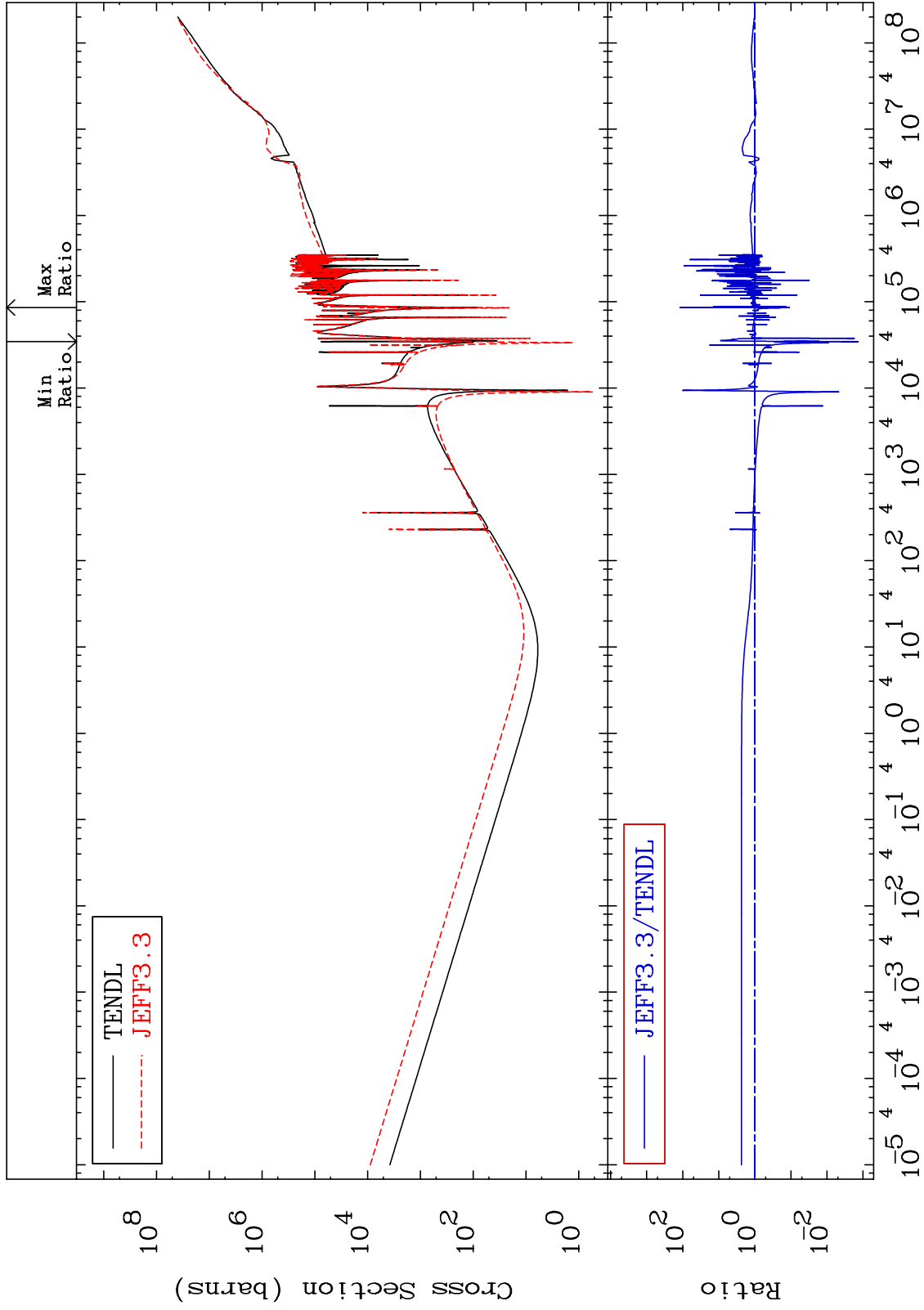
²⁶Fe-58
-99.84 To 3734. %



MAT 2637

Kerma total (eV-barns)
Cross Section

26-Fe-58
-99.87 To 9999. %



42

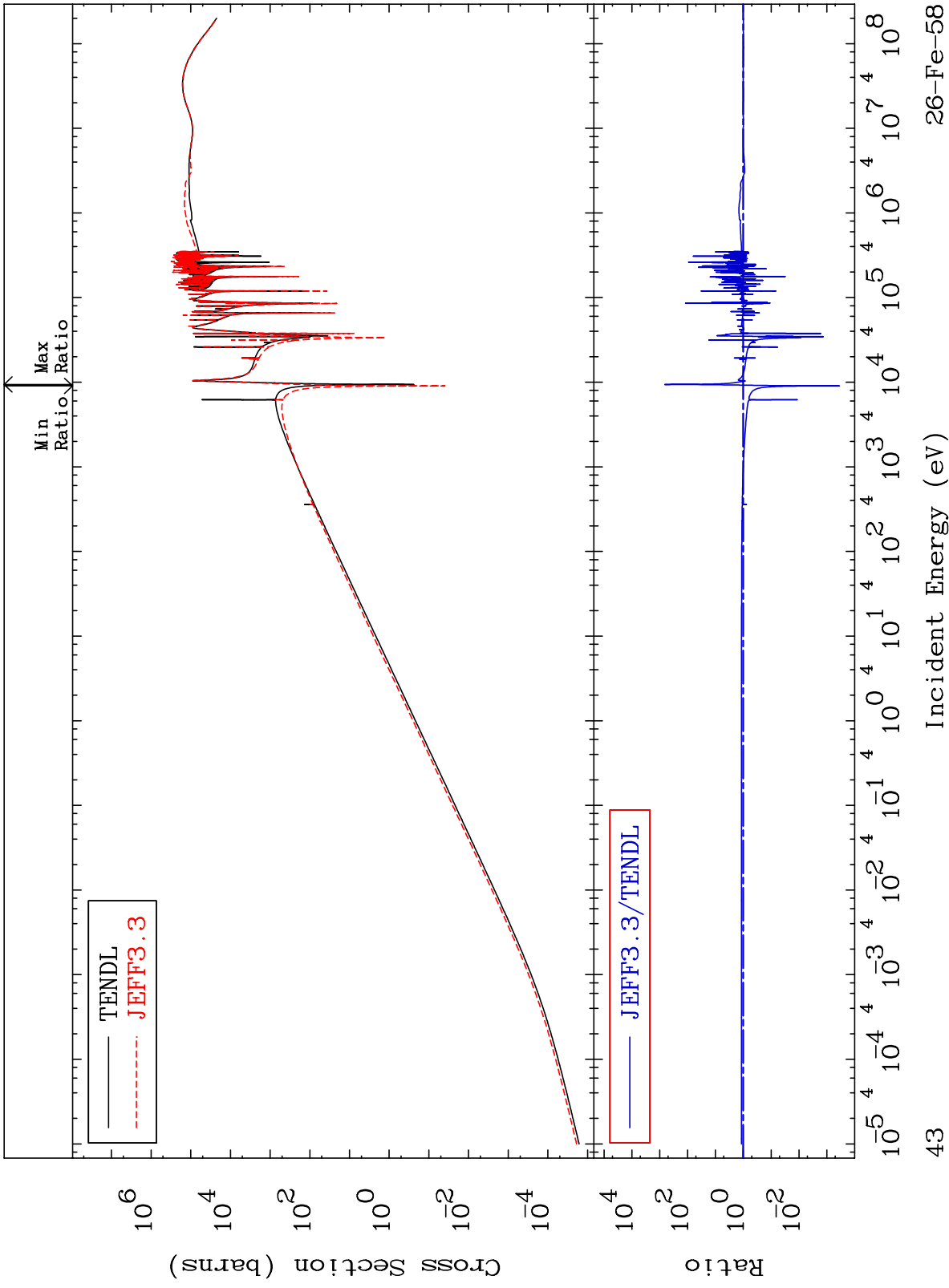
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma elastic
Cross Section

26-Fe-58
-99.97 To 9999. %



43

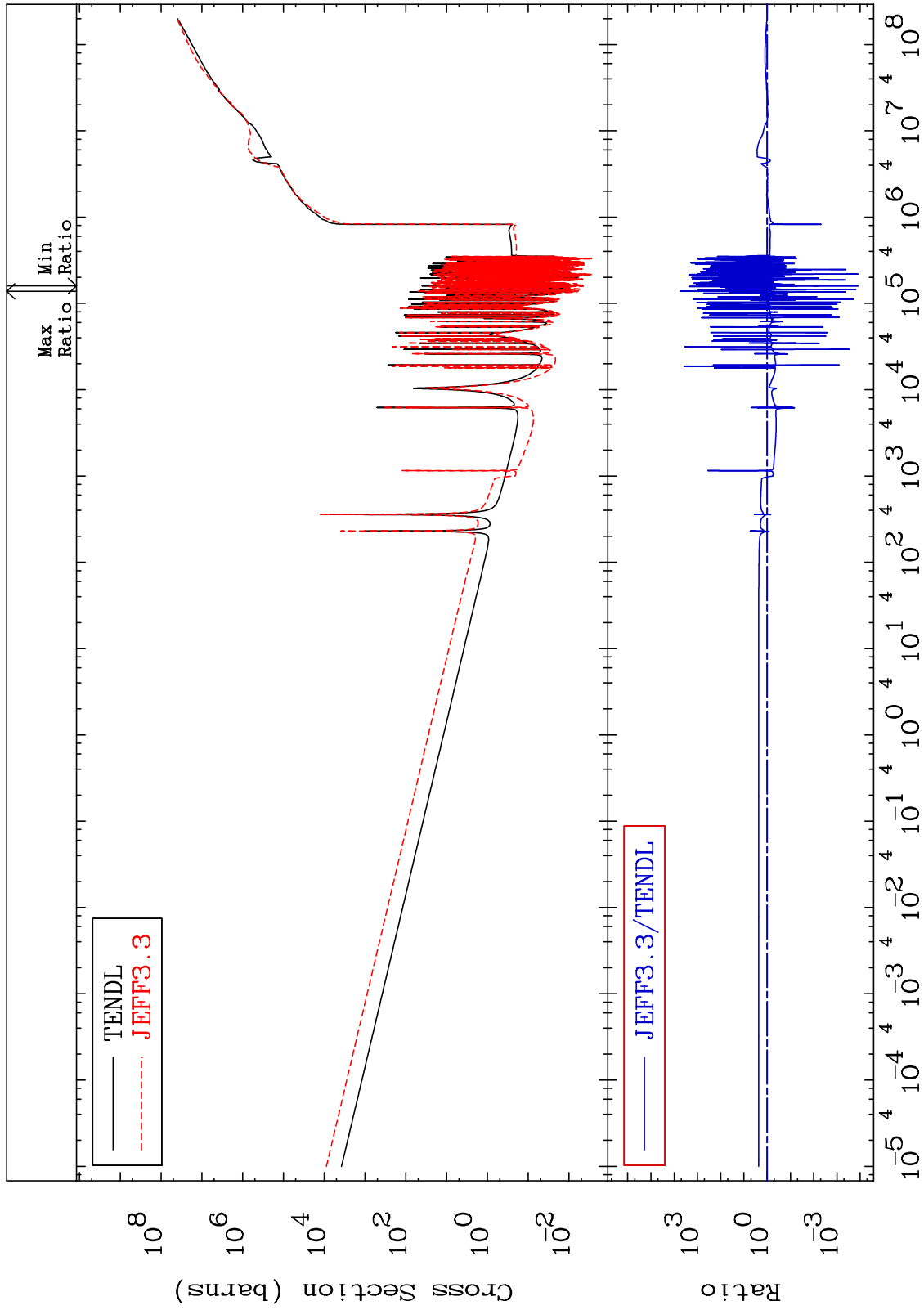
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma non-elastic (all but mt2)
Cross Section

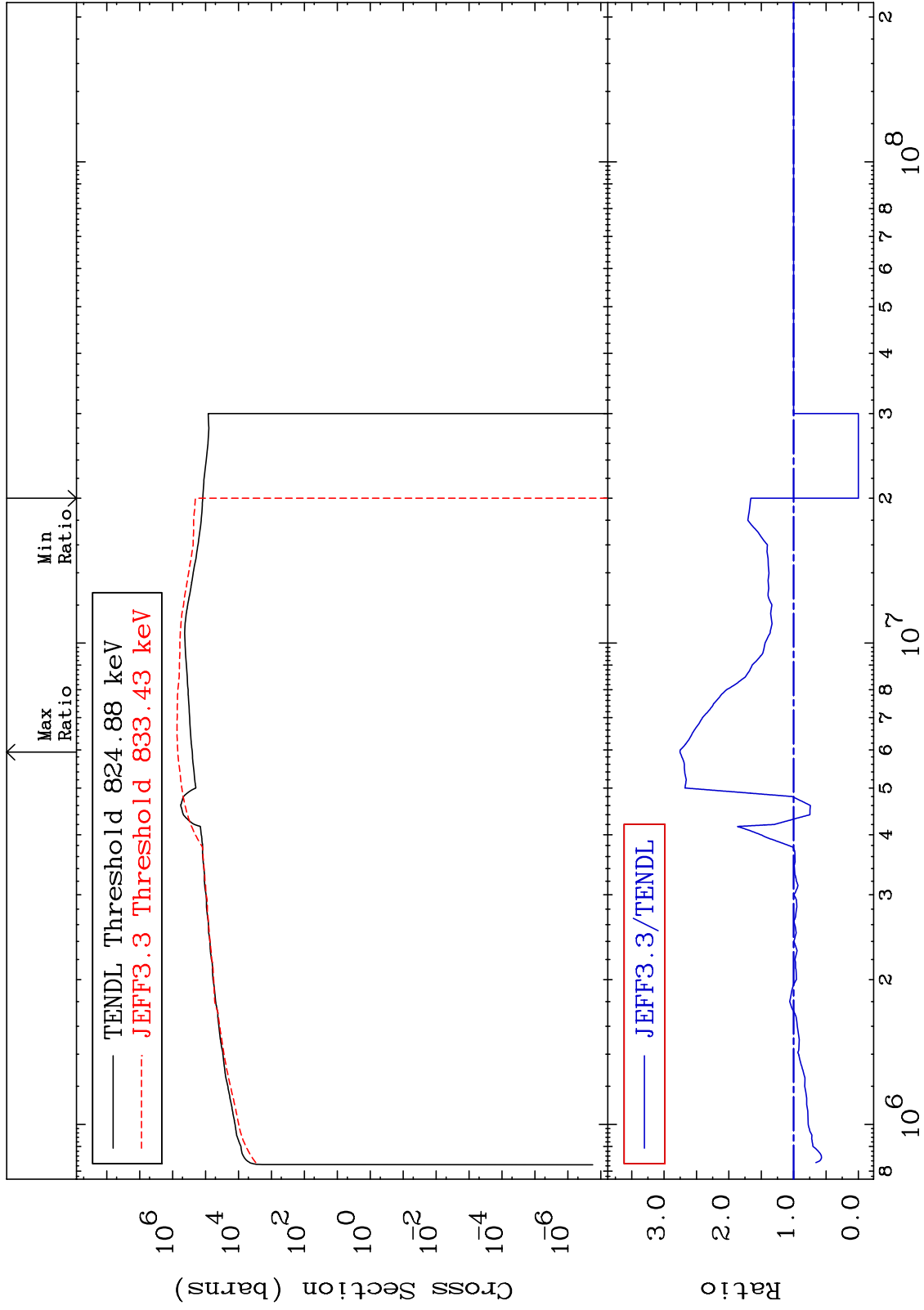
26-Fe-58
-99.99 To 9999. %



MAT 2637

Kerma inelastic (mt51-91)
Cross Section

26-Fe-58
-100.0 To 175.4 %



45

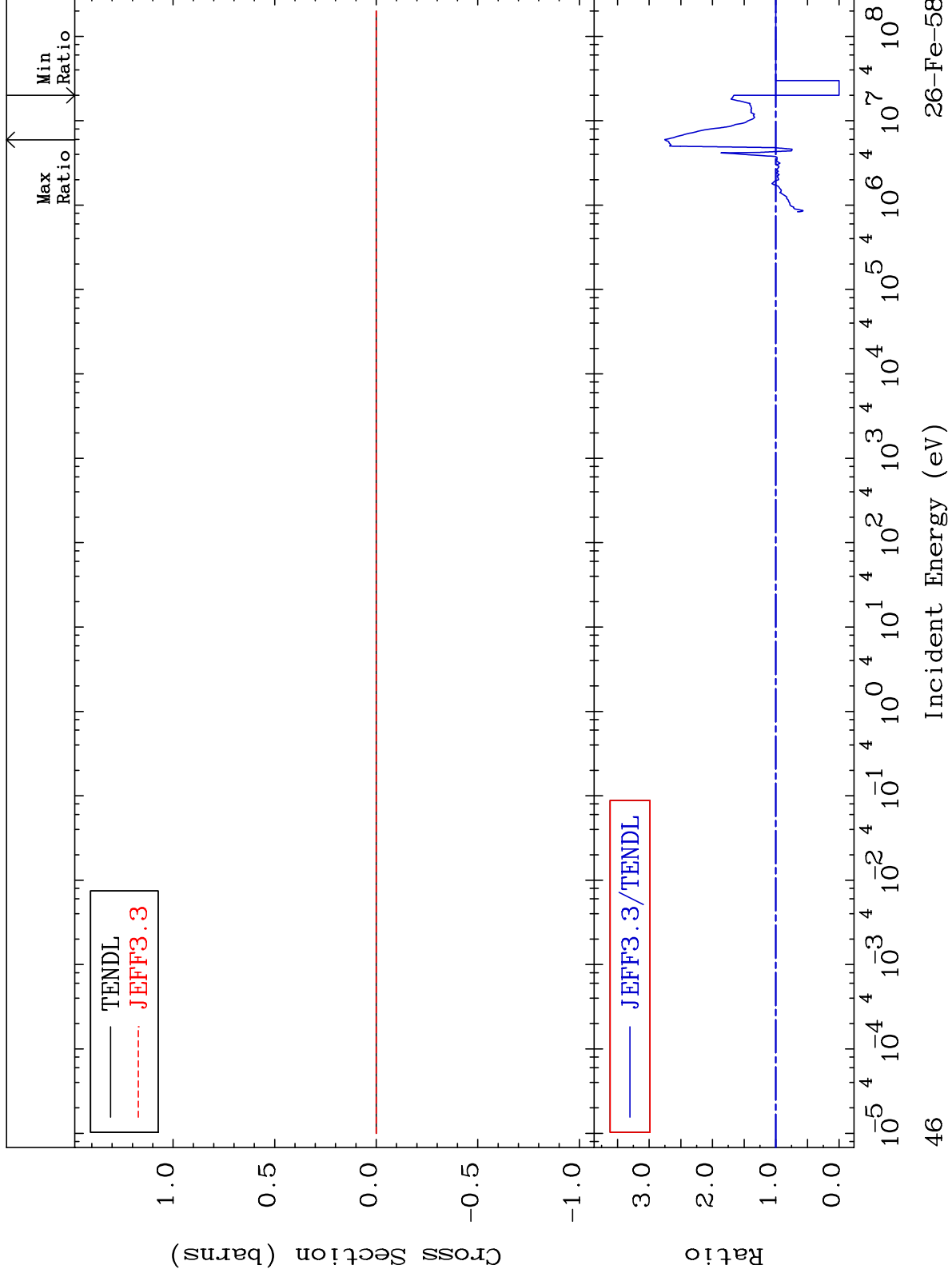
Incident Energy (eV)

26-Fe-58

MAT 2637

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

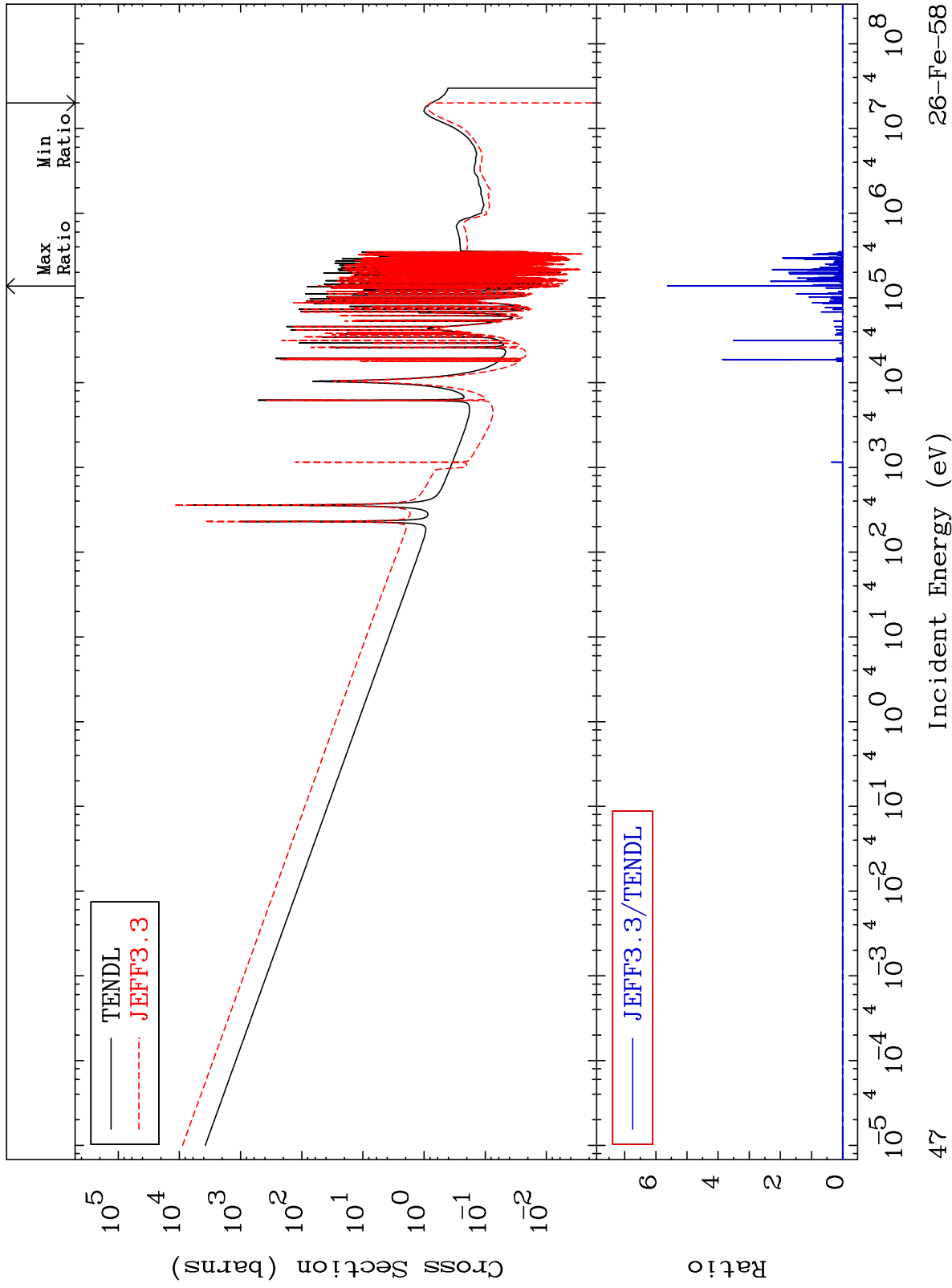
26-Fe-58
-100.0 To 175.4 %



MAT 2637

Kerma capture (mt102)
Cross Section

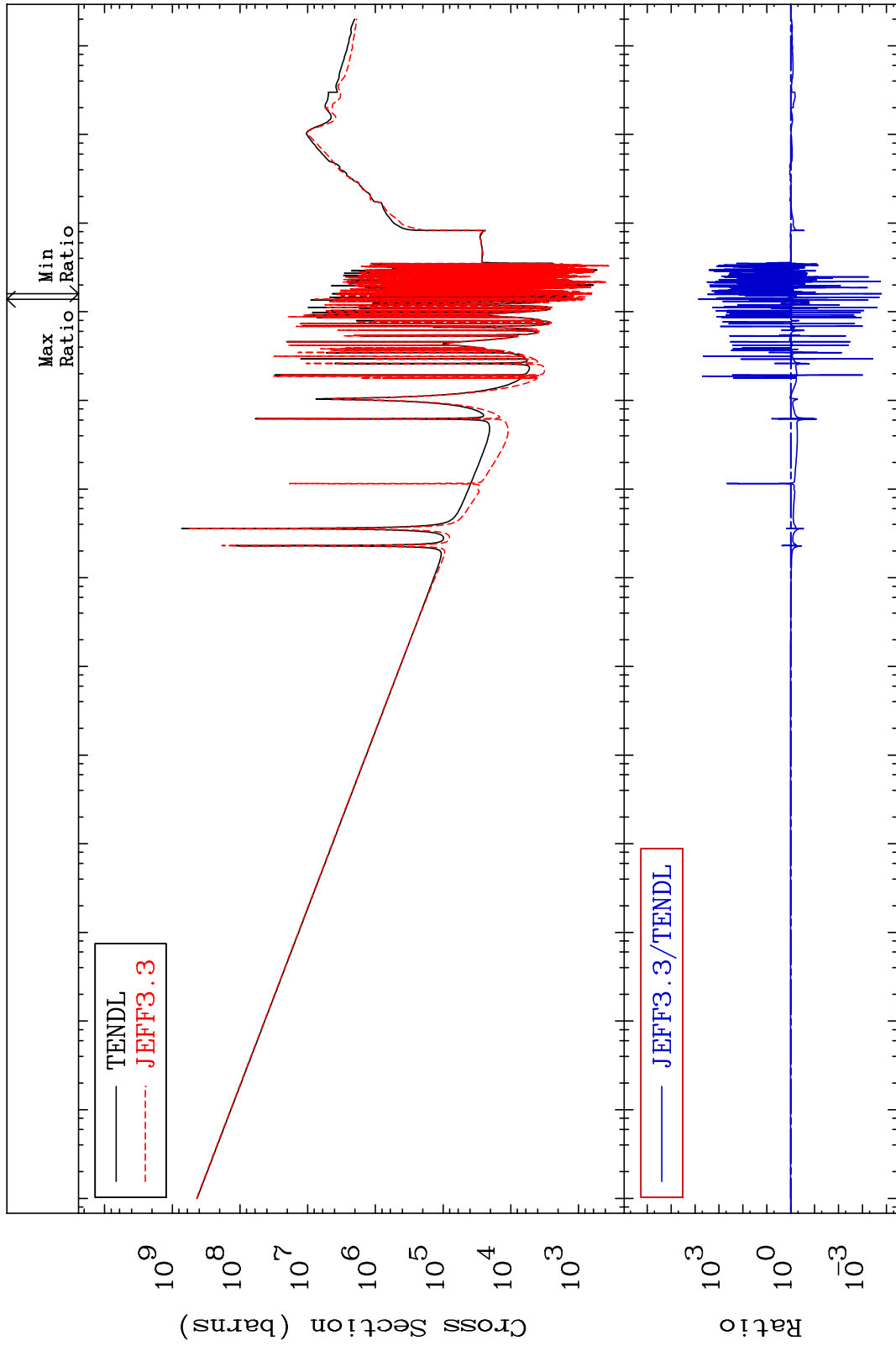
26-Fe-58
-100.0 To 9999. %



MAT 2637

Total photon (eV-barns)
Cross Section

26-Fe-58
-99.98 To 9999. %



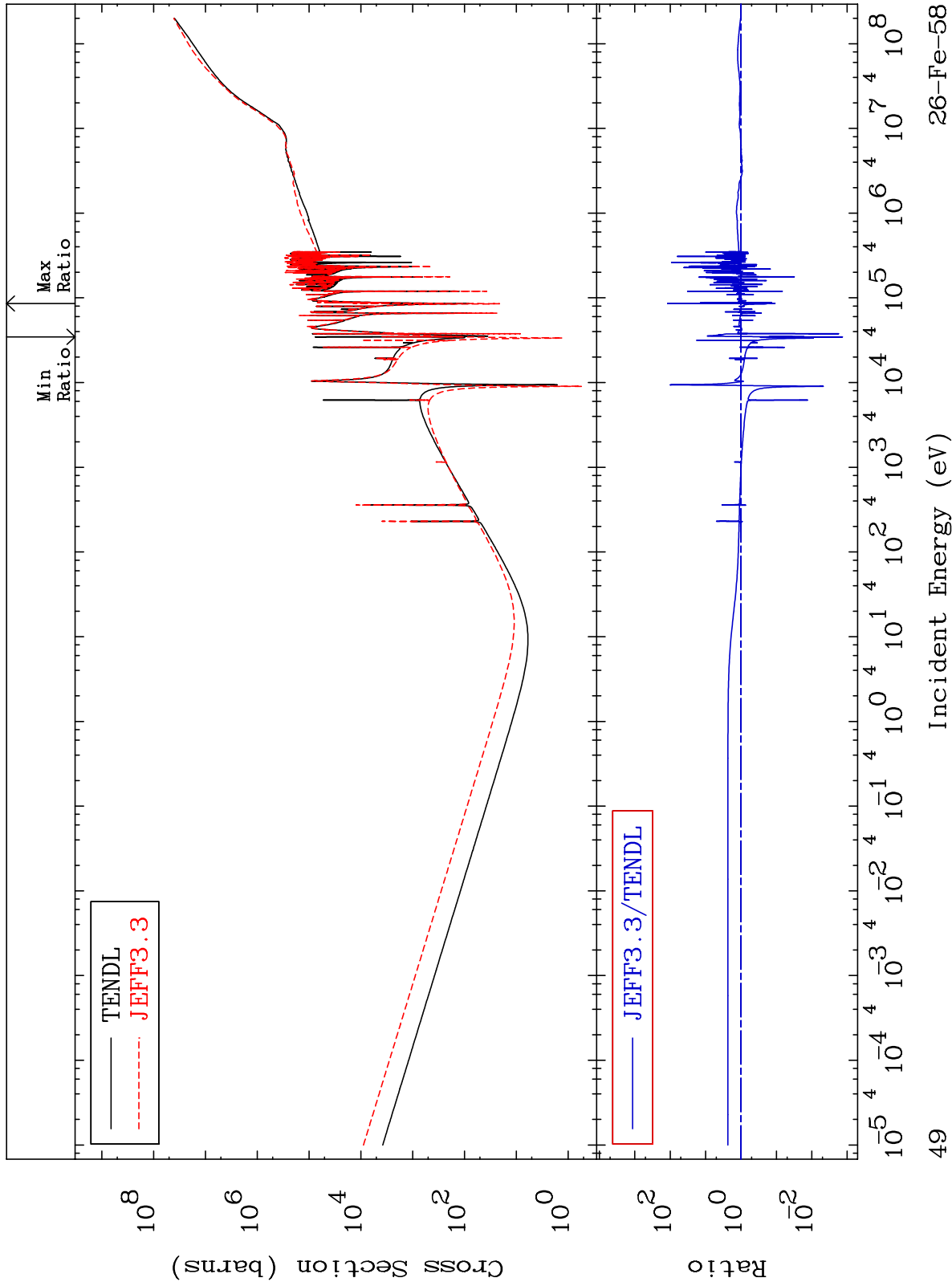
— TENDL
- - - JEFF3.3

— JEFF3.3/TENDL

MAT 2637

Total kinematic kerma (high limit)
Cross Section

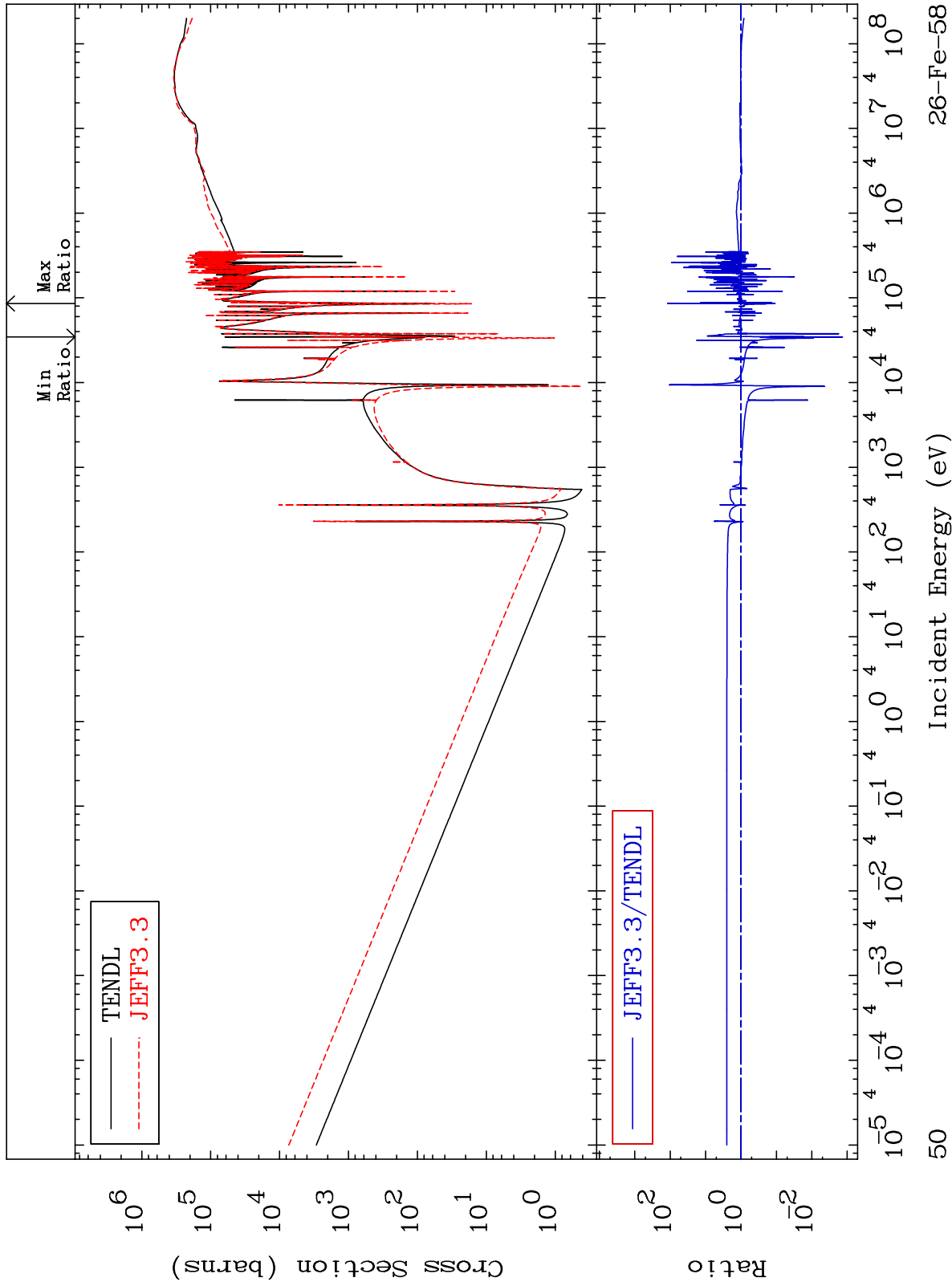
26-Fe-58
-99.87 To 9999. %



MAT 2637

Dpa total (eV-barns)
Cross Section

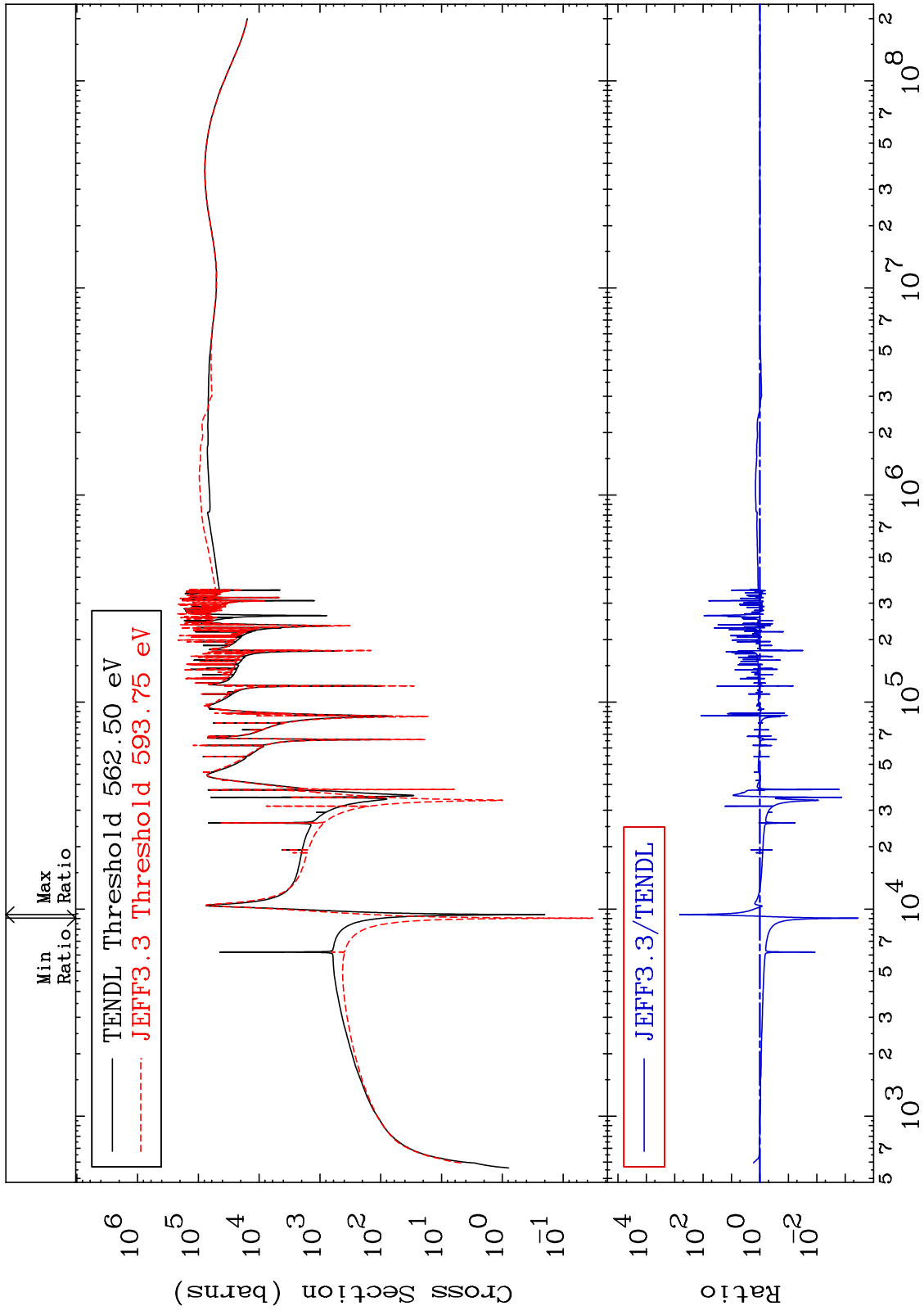
26-Fe-58
-99.87 To 9999. %



MAT 2637

Dpa elastic (mt2)
Cross Section

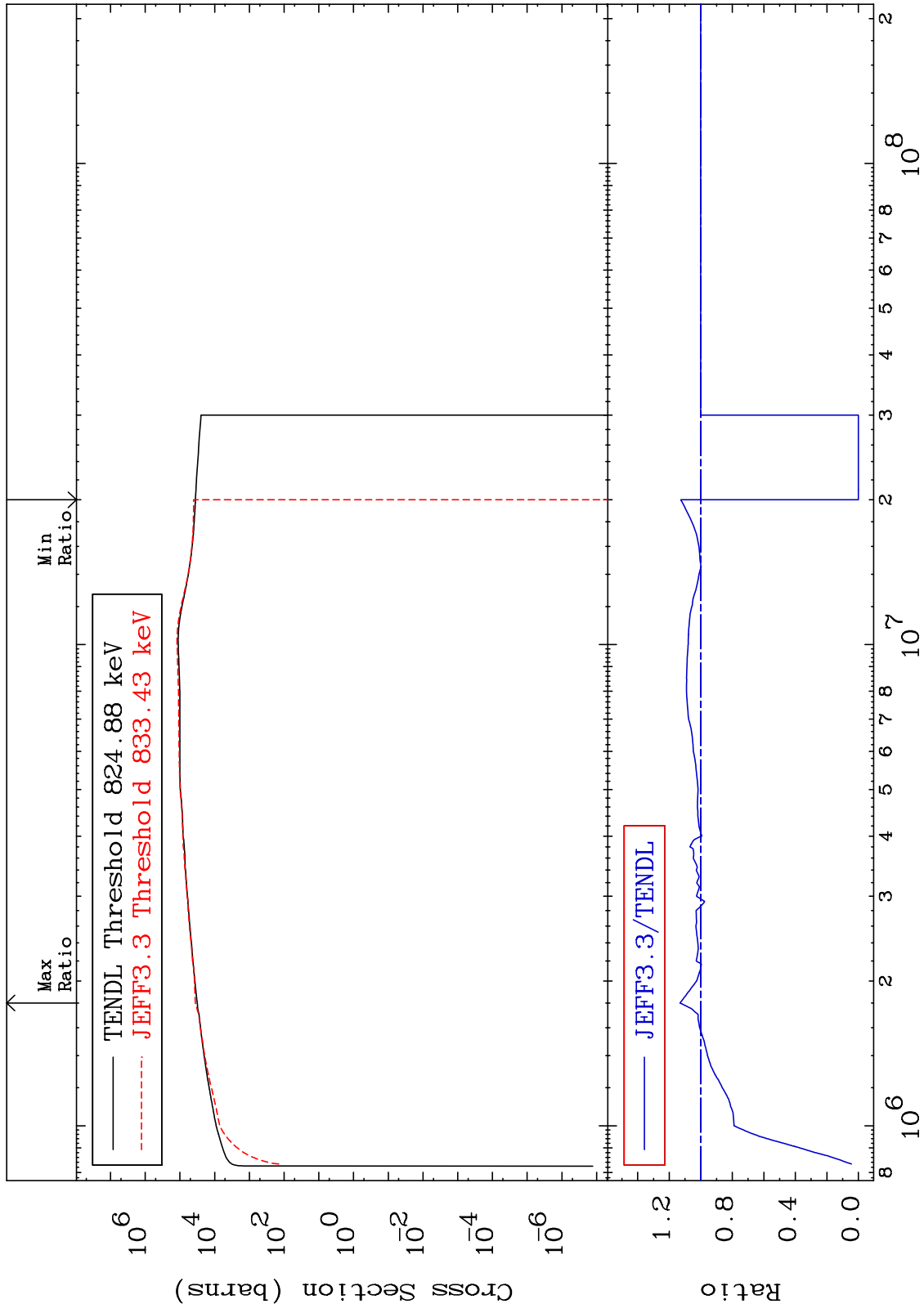
26-Fe-58
-99.97 To 9999. %



MAT 2637

Dpa inelastic (mt51-91)
Cross Section

²⁶Fe-58
-100.0 To 13.16 %



52

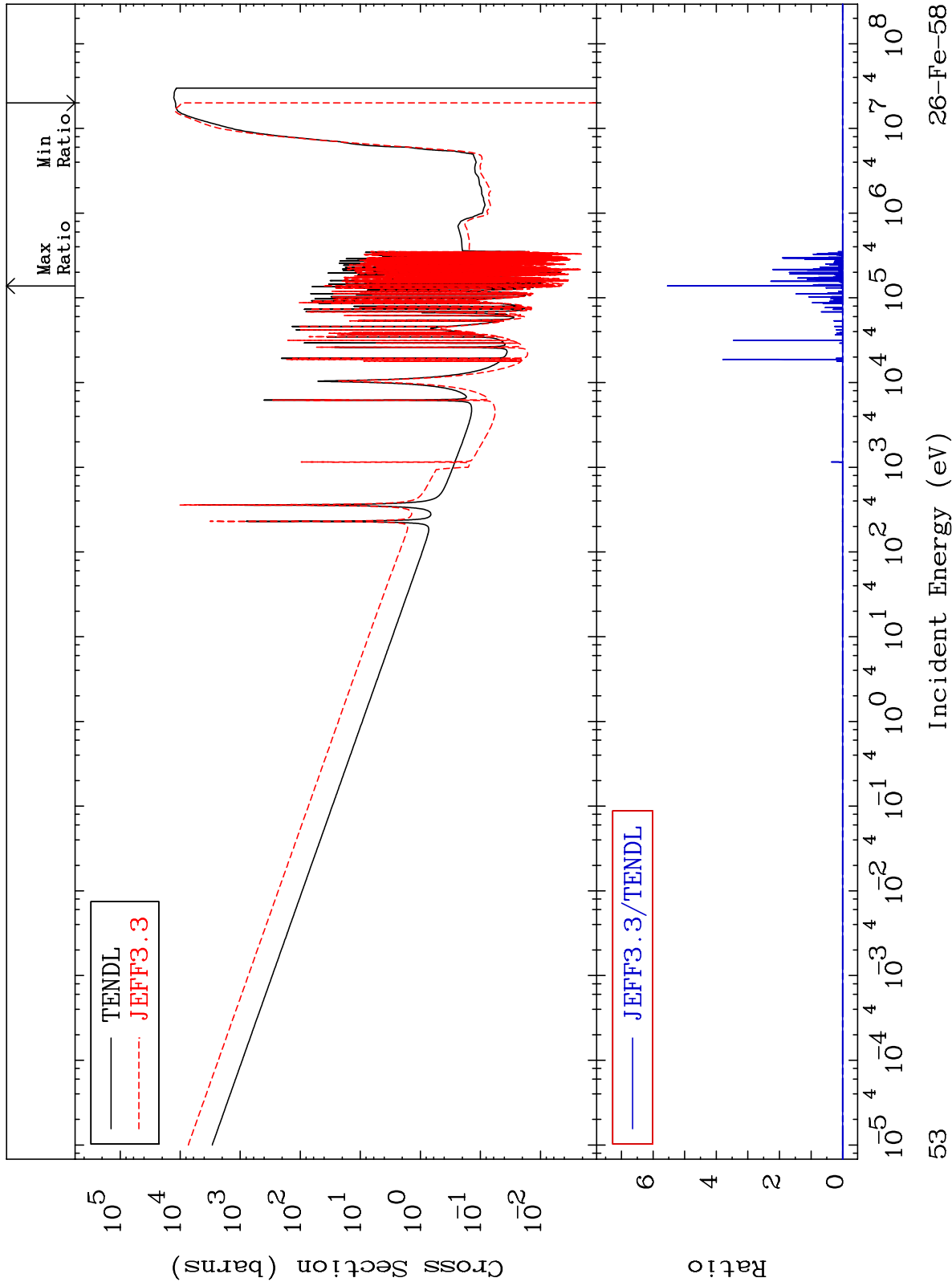
Incident Energy (eV)

²⁶Fe-58

MAT 2637

Dpa disappearance (mt102 -120)
Cross Section

26-Fe-58
-100.0 To 9999. %



53

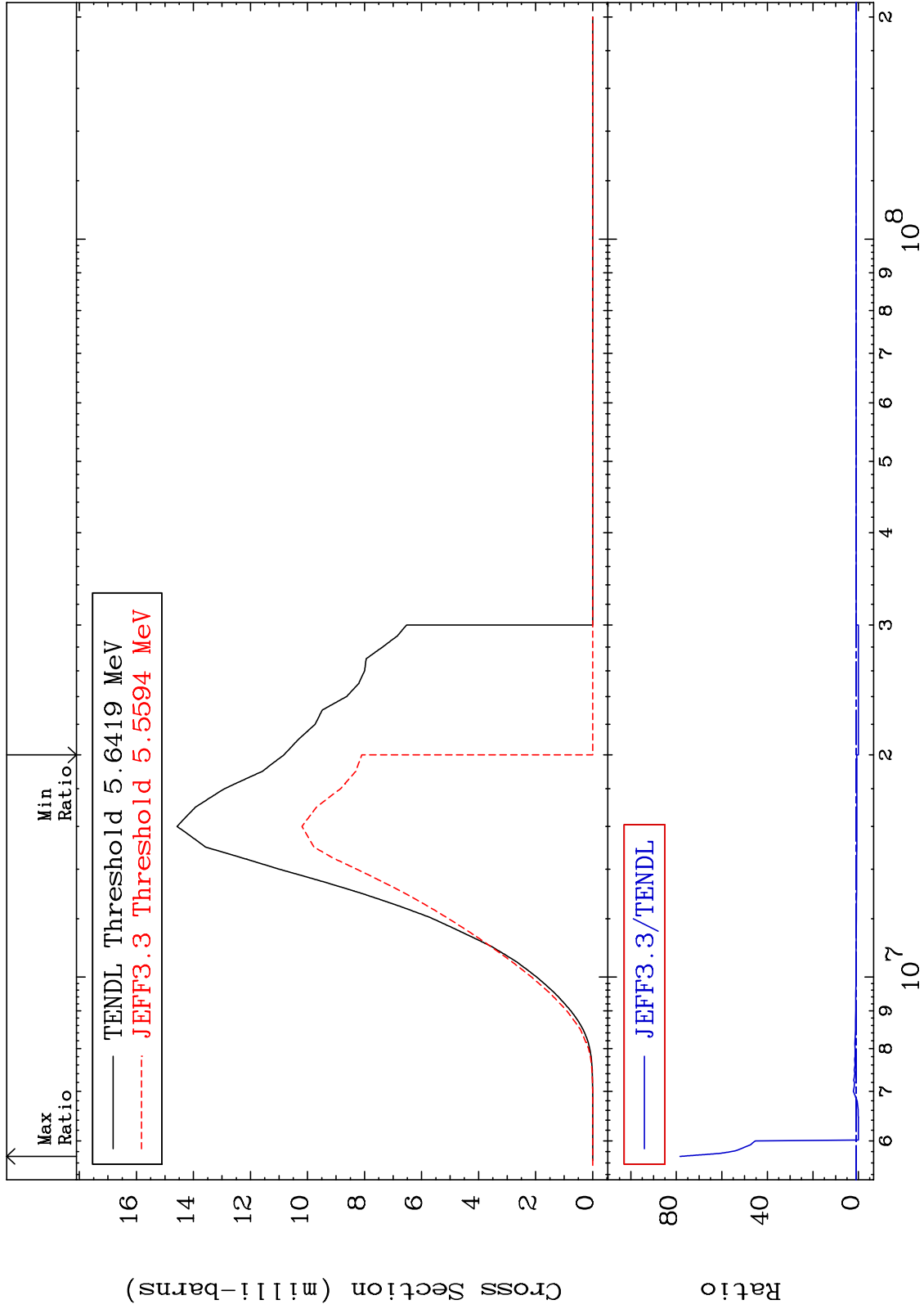
26-Fe-58

MAT 2637

(n,p):25-Mn-58g

26-Fe-58

Radionuclide Production Cross Section -100.0 To 7738. %

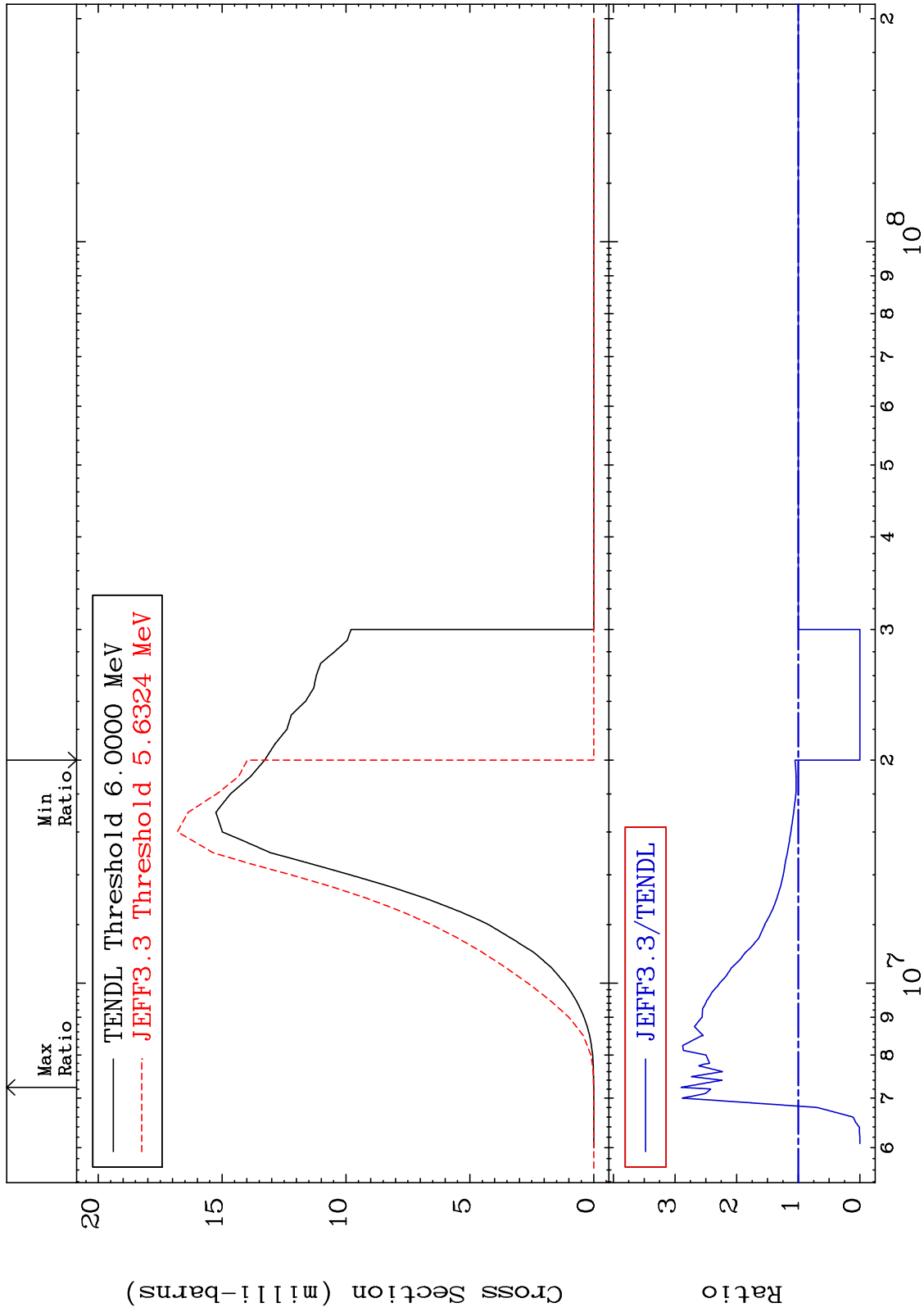


MAT 2637

(n, p):25-Mn-58m1

26-Fe-58

Radionuclide Production Cross Section -100.0 To 189.9 %



55

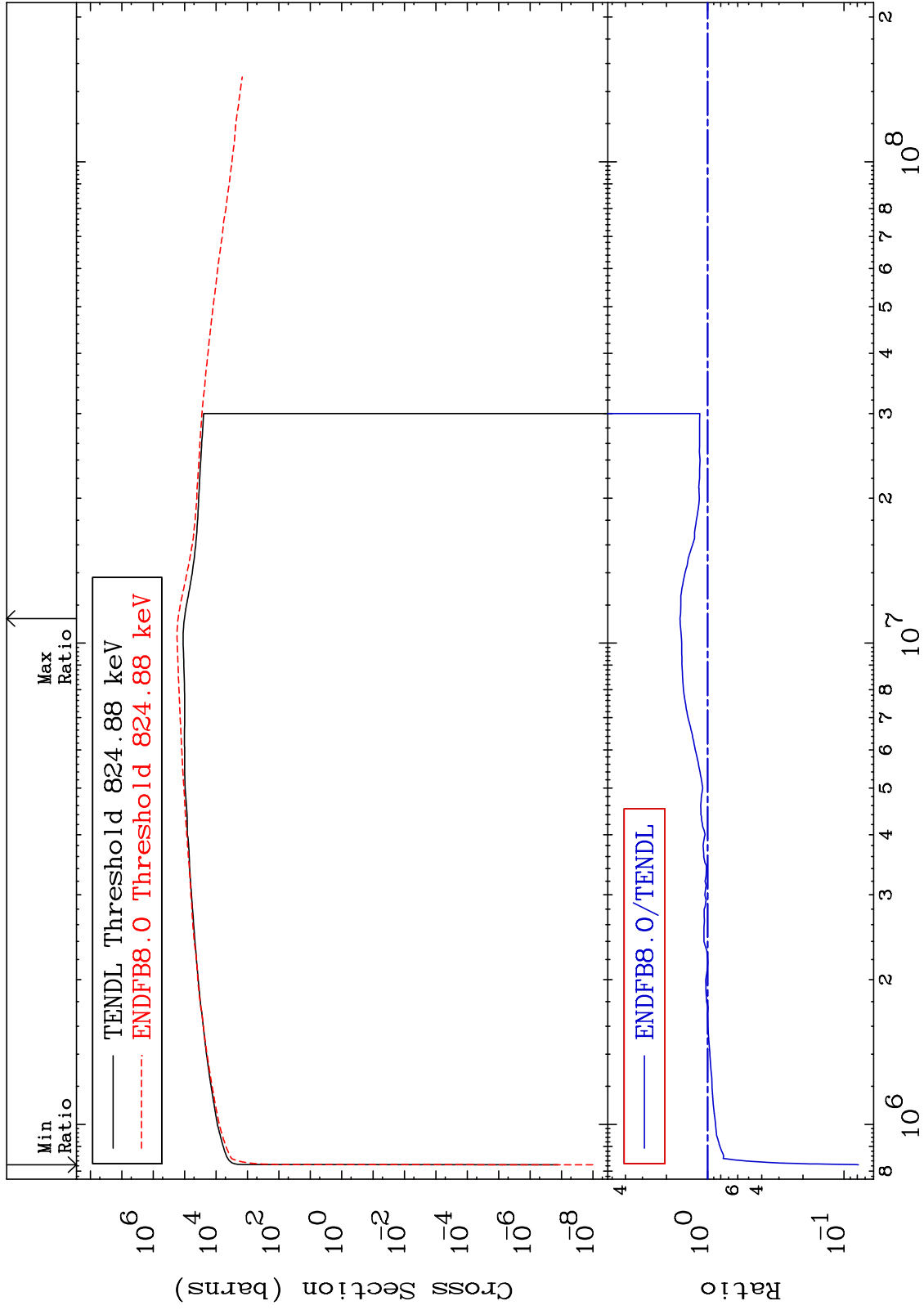
Incident Energy (eV)

26-Fe-58

MAT 2637

Dpa inelastic (mt51-91)
Cross Section

²⁶Fe-58
-92.16 To 59.19 %



56

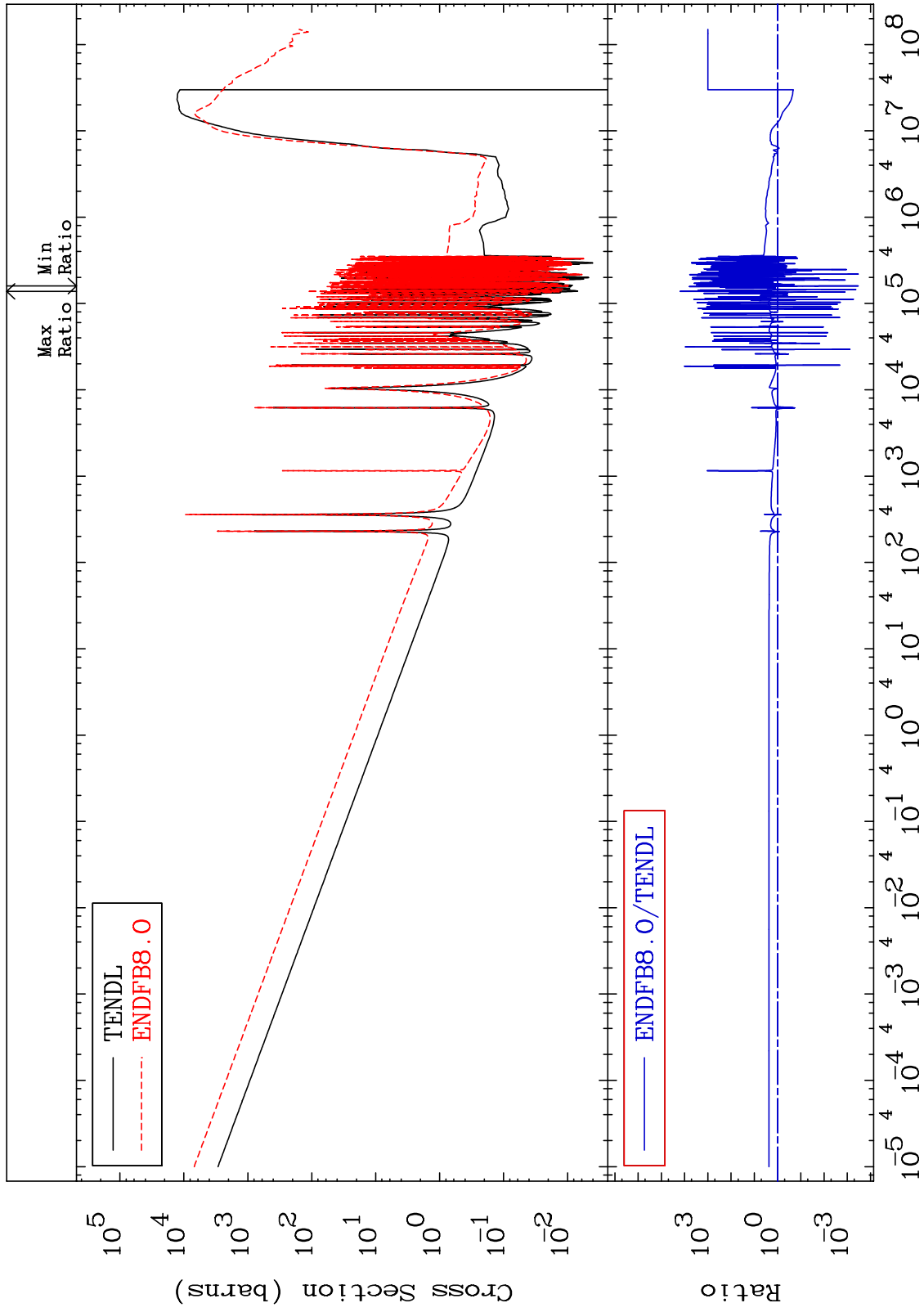
26-Fe-58

26-Fe-58

MAT 2637

Dpa disappearance (mt102 -120)
Cross Section

26-Fe-58
-99.97 To 9999. %



57

Incident Energy (eV)

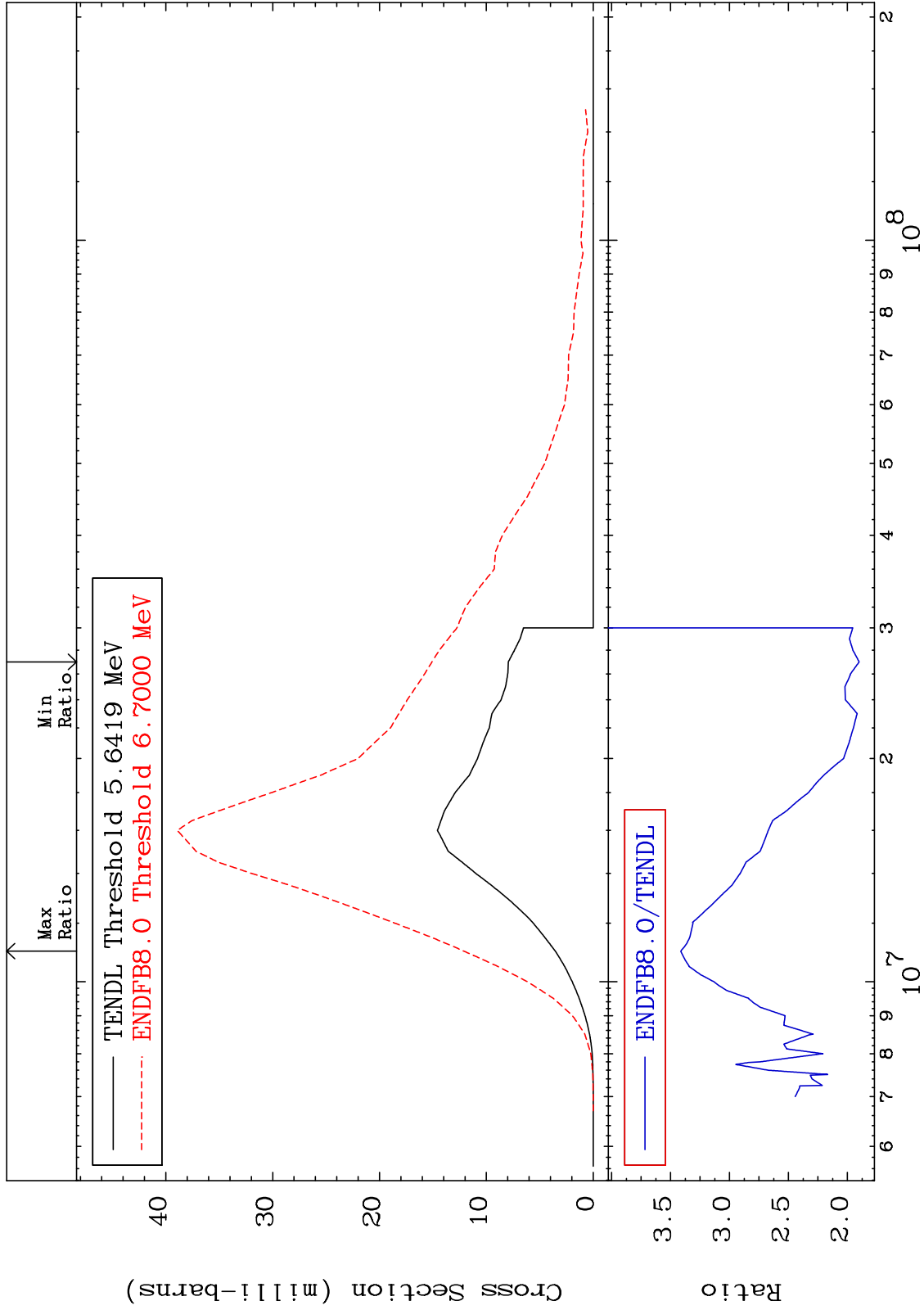
26-Fe-58

MAT 2637

(n, p) : 25-Mn-58g

26-Fe-58

Radionuclide Production Cross Section 90.01 To 241.4 %



58

Incident Energy (eV)

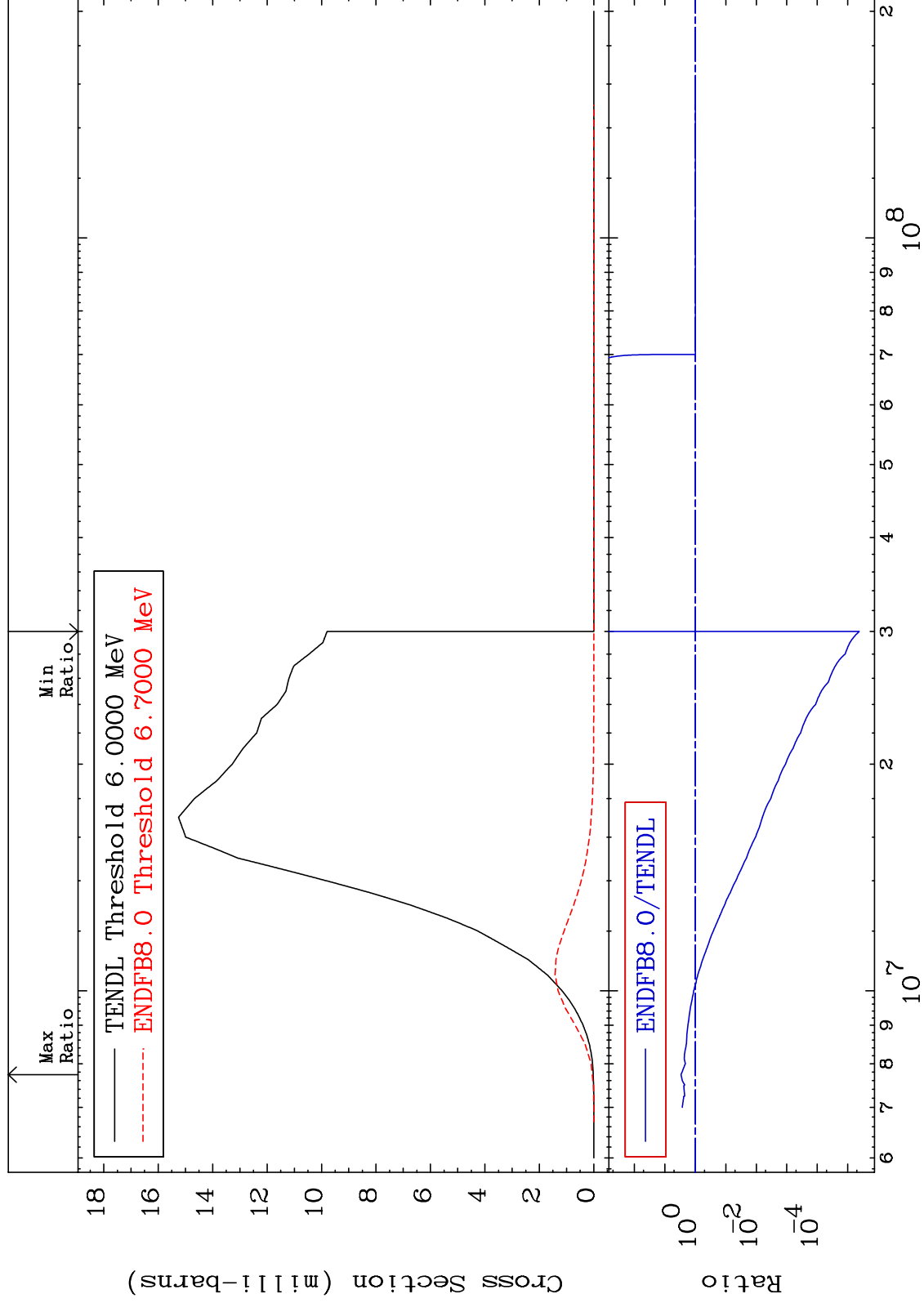
26-Fe-58

MAT 2637

(n, p):25-Mn-58m1

26-Fe-58

Radionuclide Production Cross Section -100.0 To 193.2 %



59

Incident Energy (eV)

26-Fe-58