

Program Complot
(Version 2018-1)

by

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Press Mouse Button to Start

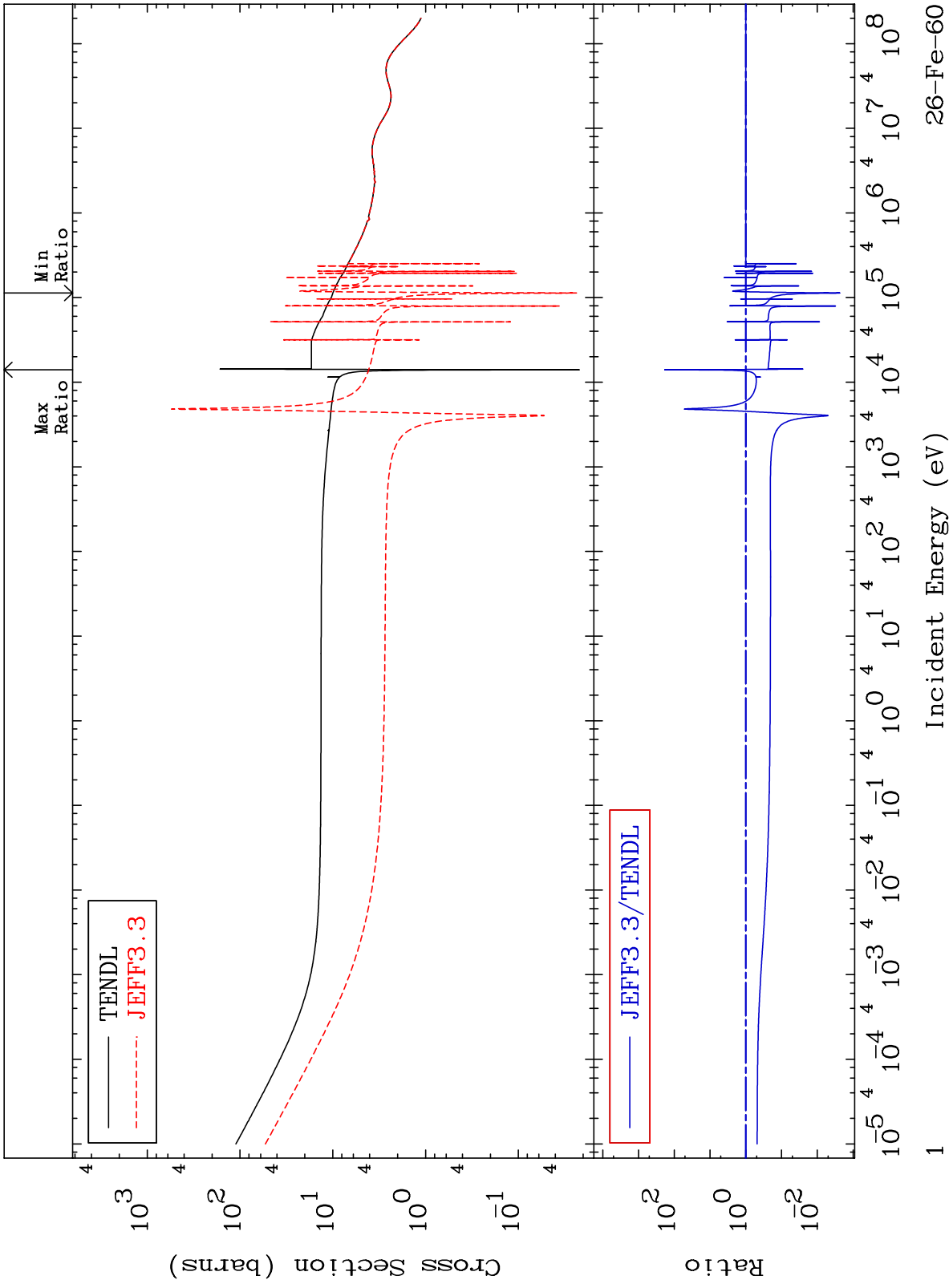
MAT 2643

Total

26-Fe-60

Cross Section

-99.76 To 9999. %



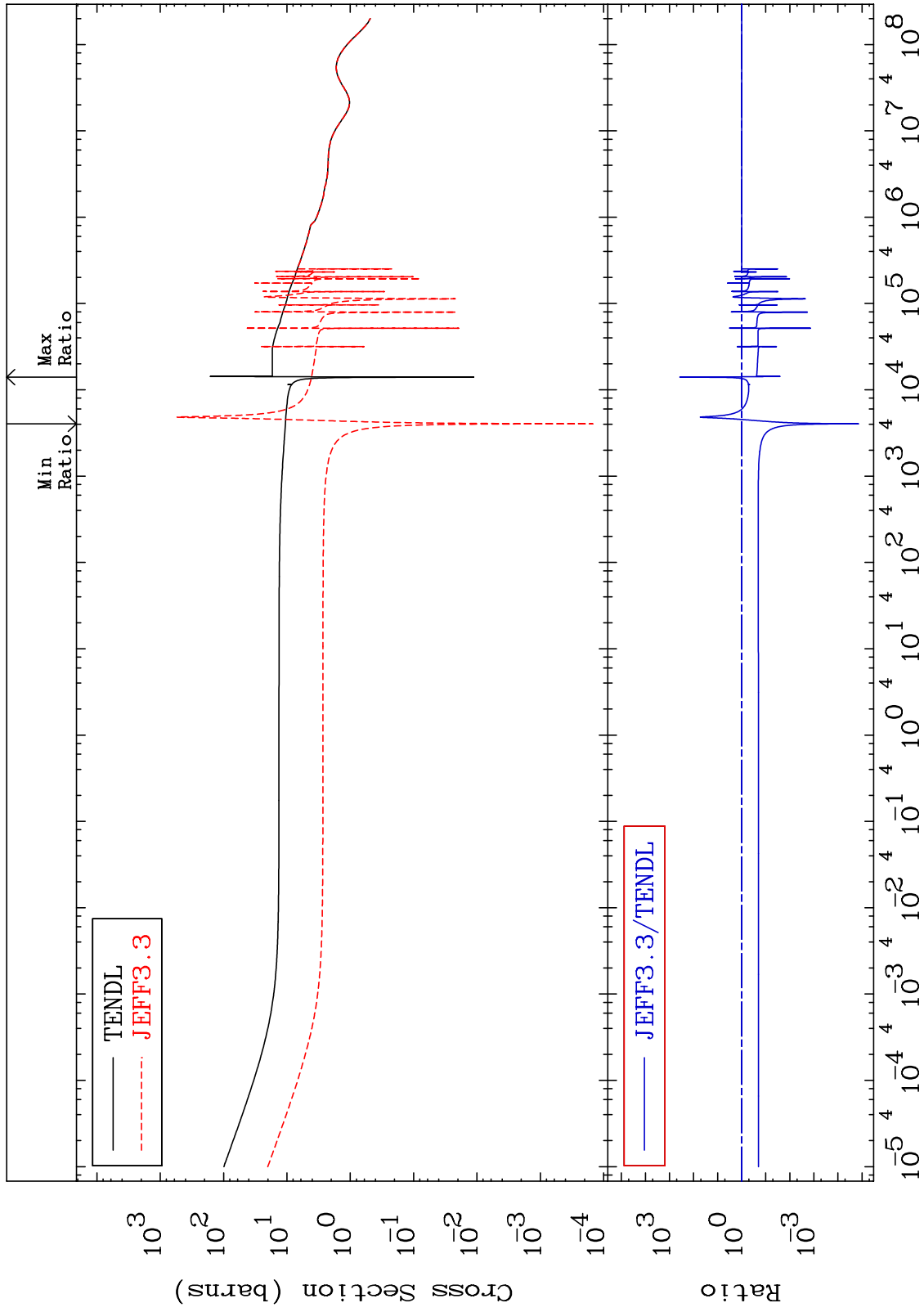
Incident Energy (eV)

26-Fe-60

MAT 2643

Elastic
Cross Section

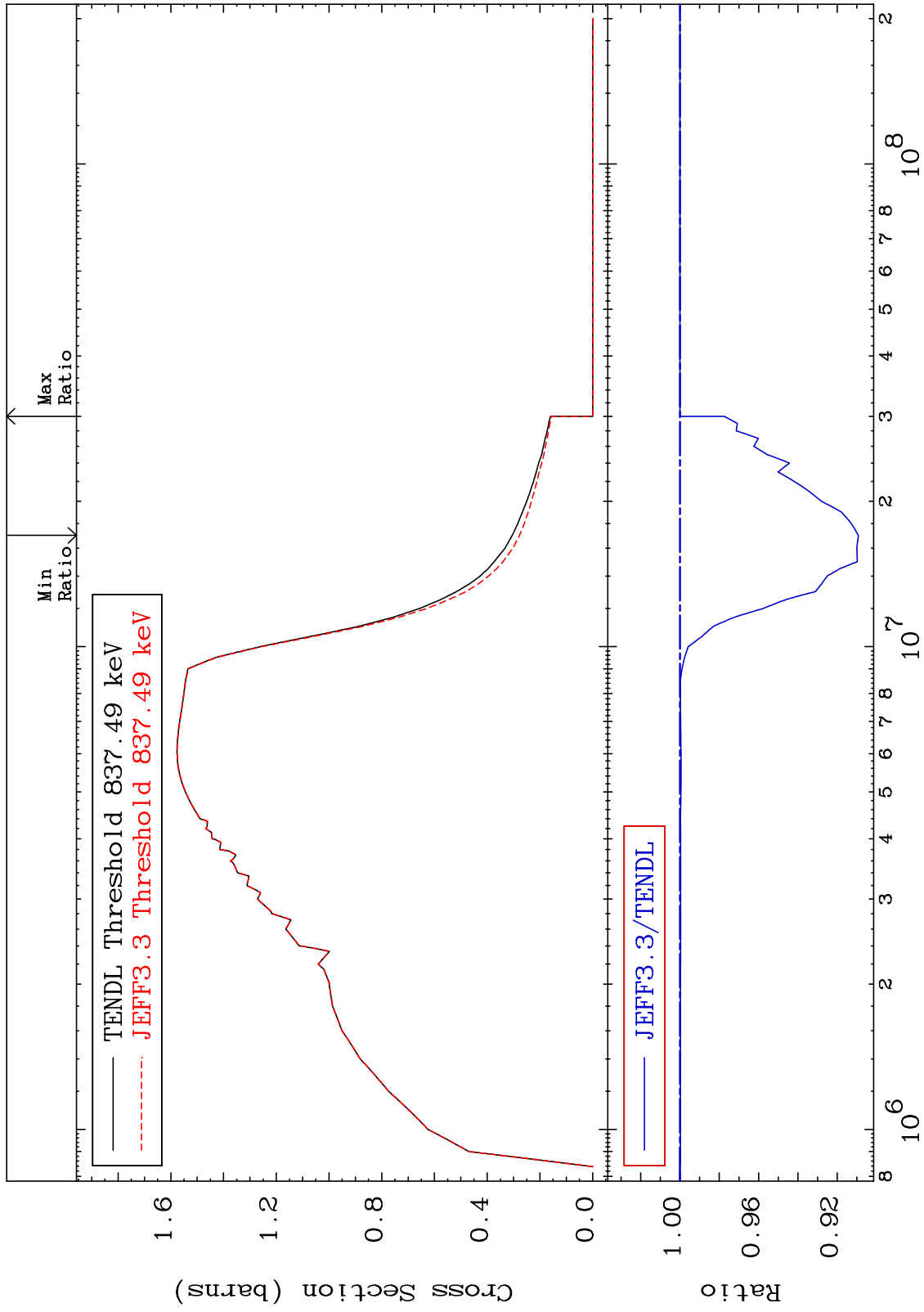
26-Fe-60
-100.0 To 9999. %



MAT 2643

Inelastic
Cross Section

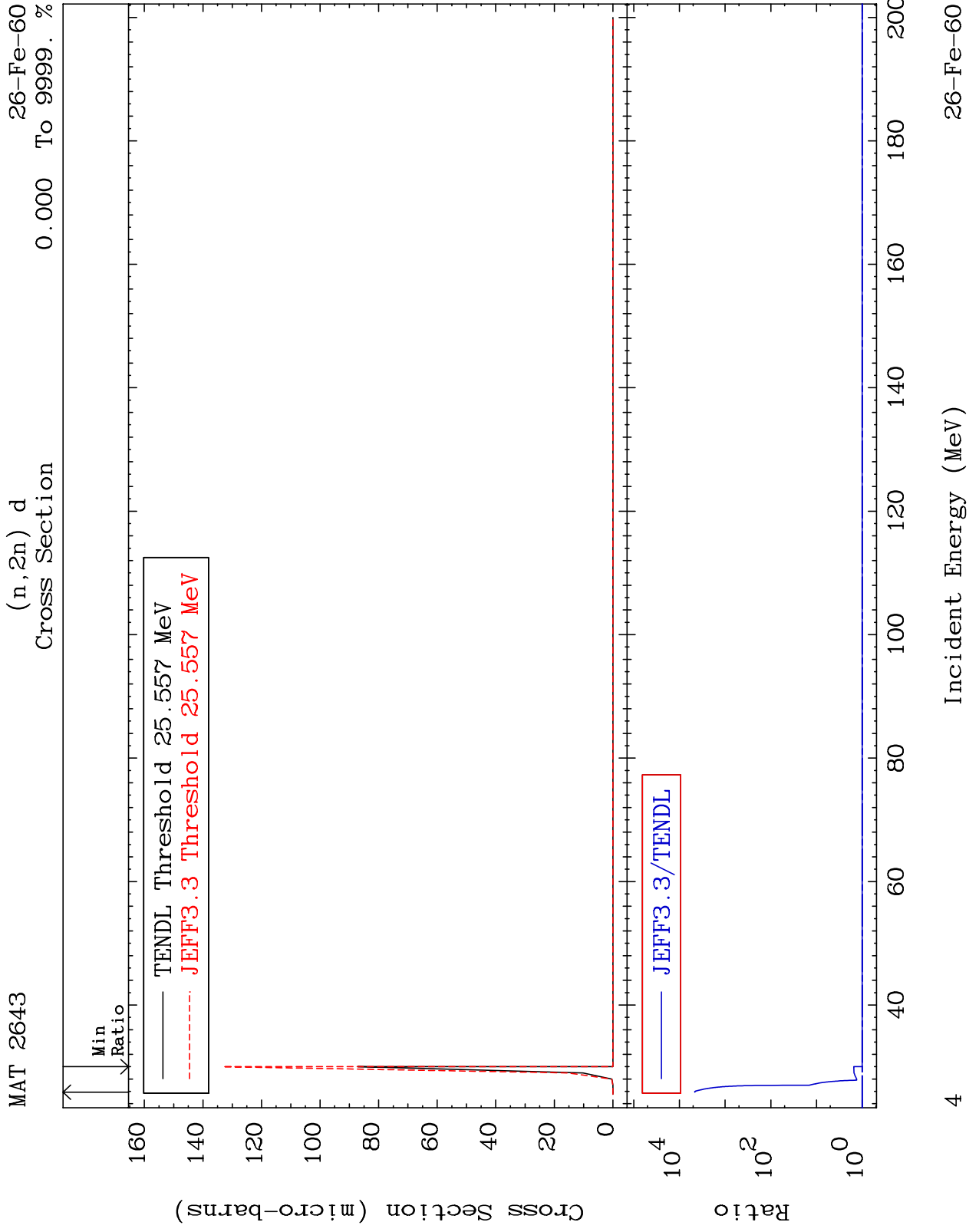
26-Fe-60
-9.081 To 0.000 %

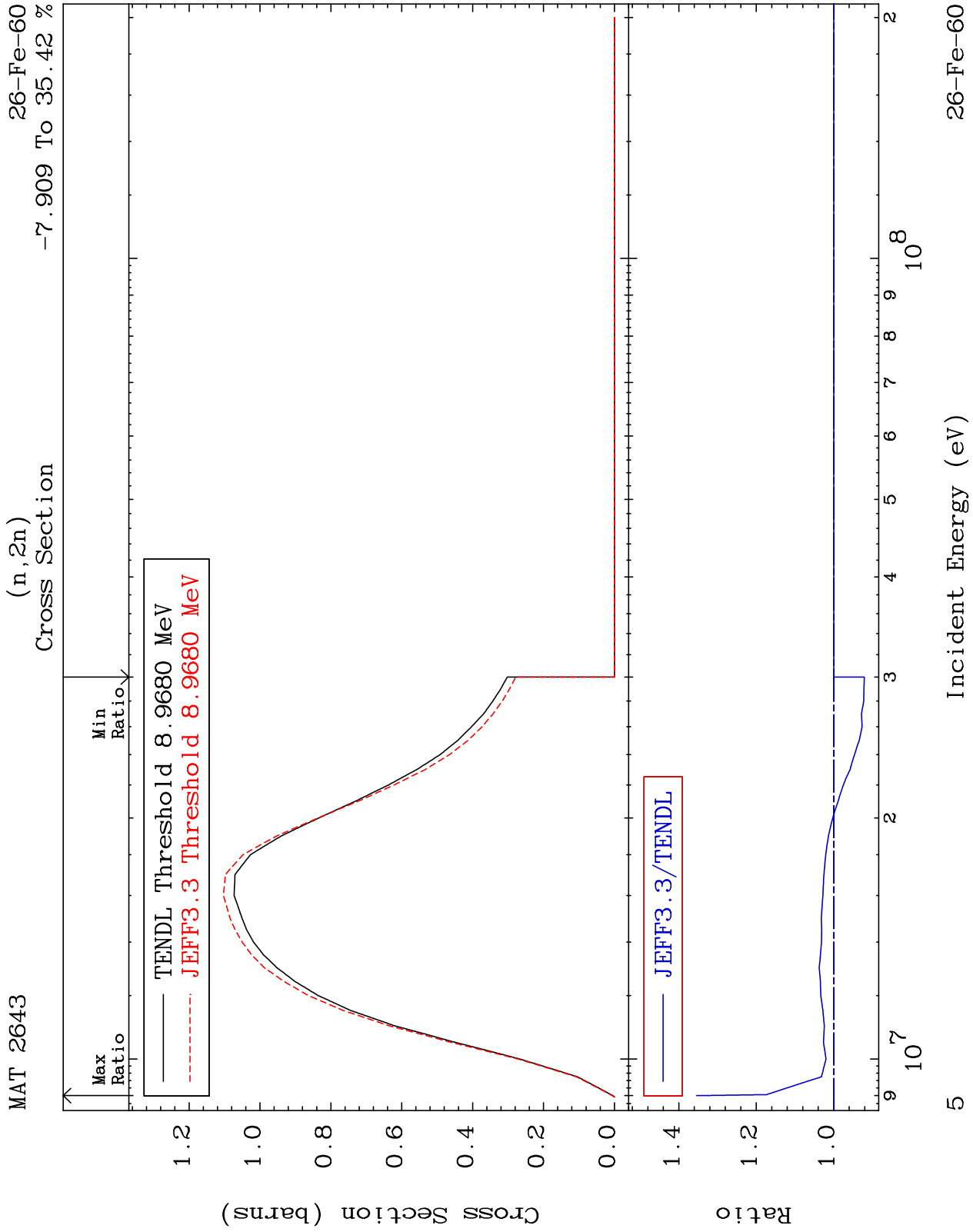


Incident Energy (eV)

26-Fe-60

3





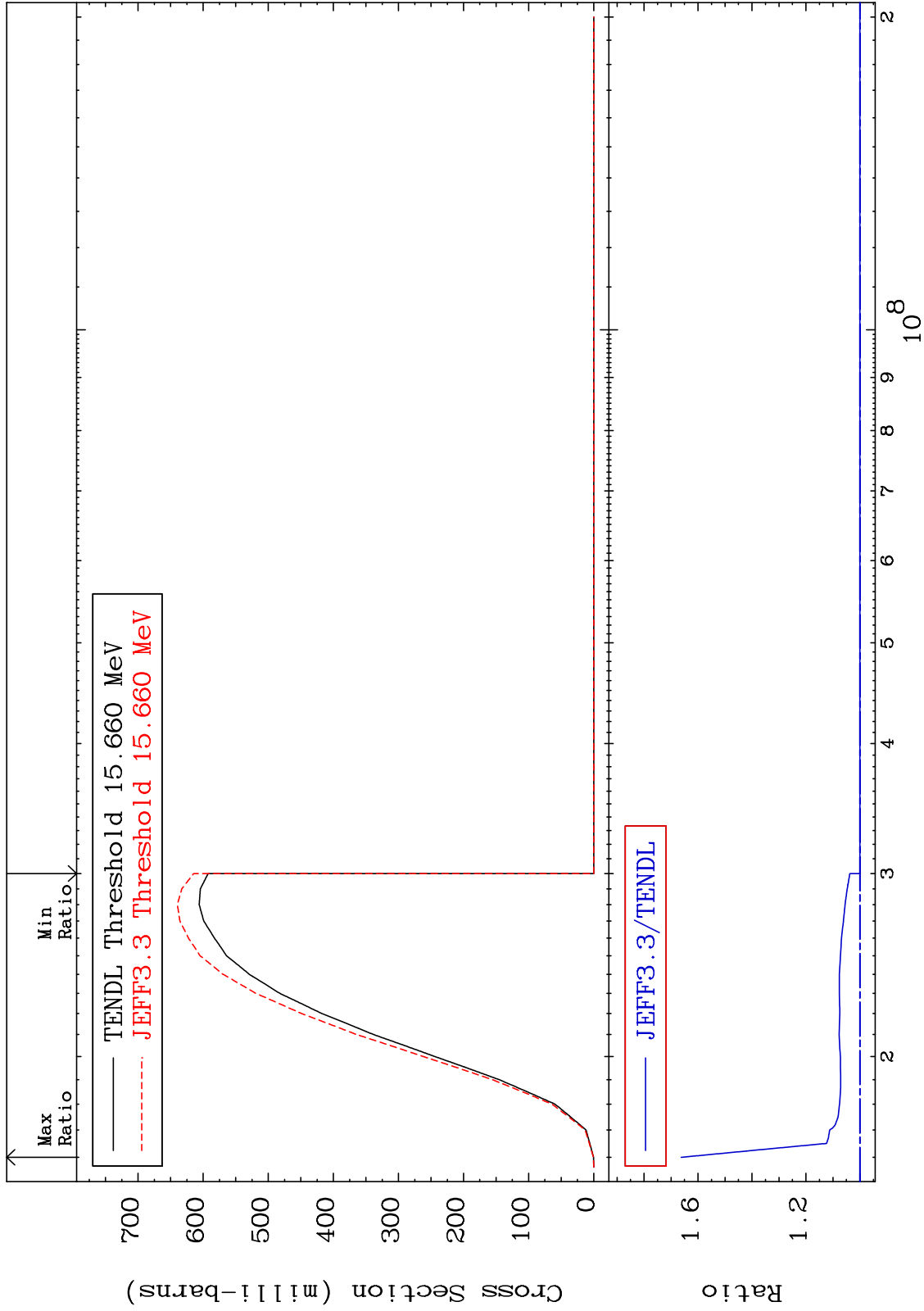
MAT 2643

(n, 3n)

²⁶Fe-60

Cross Section

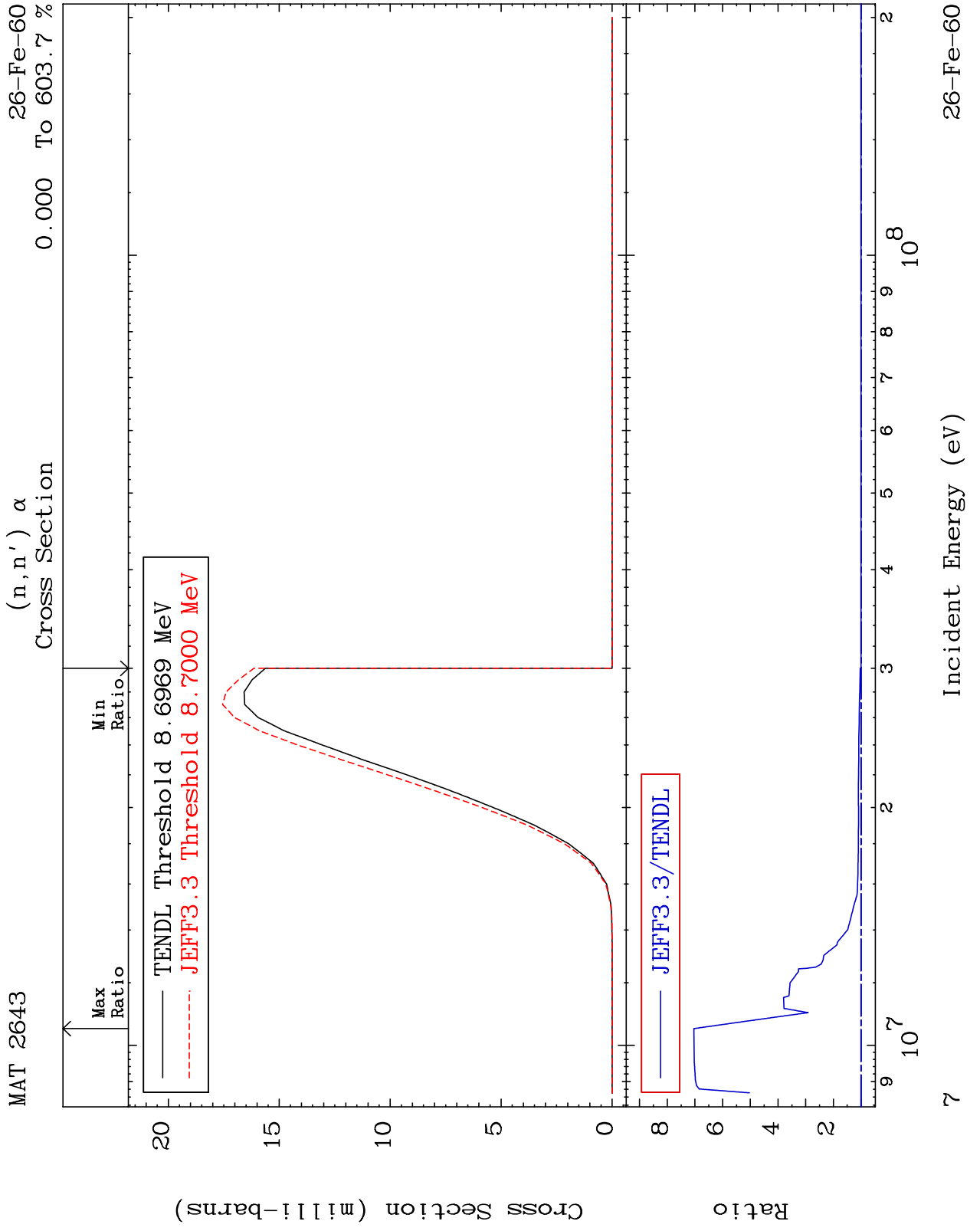
0.000 To 66.23 %



6

Incident Energy (eV)

²⁶Fe-60



MAT 2643

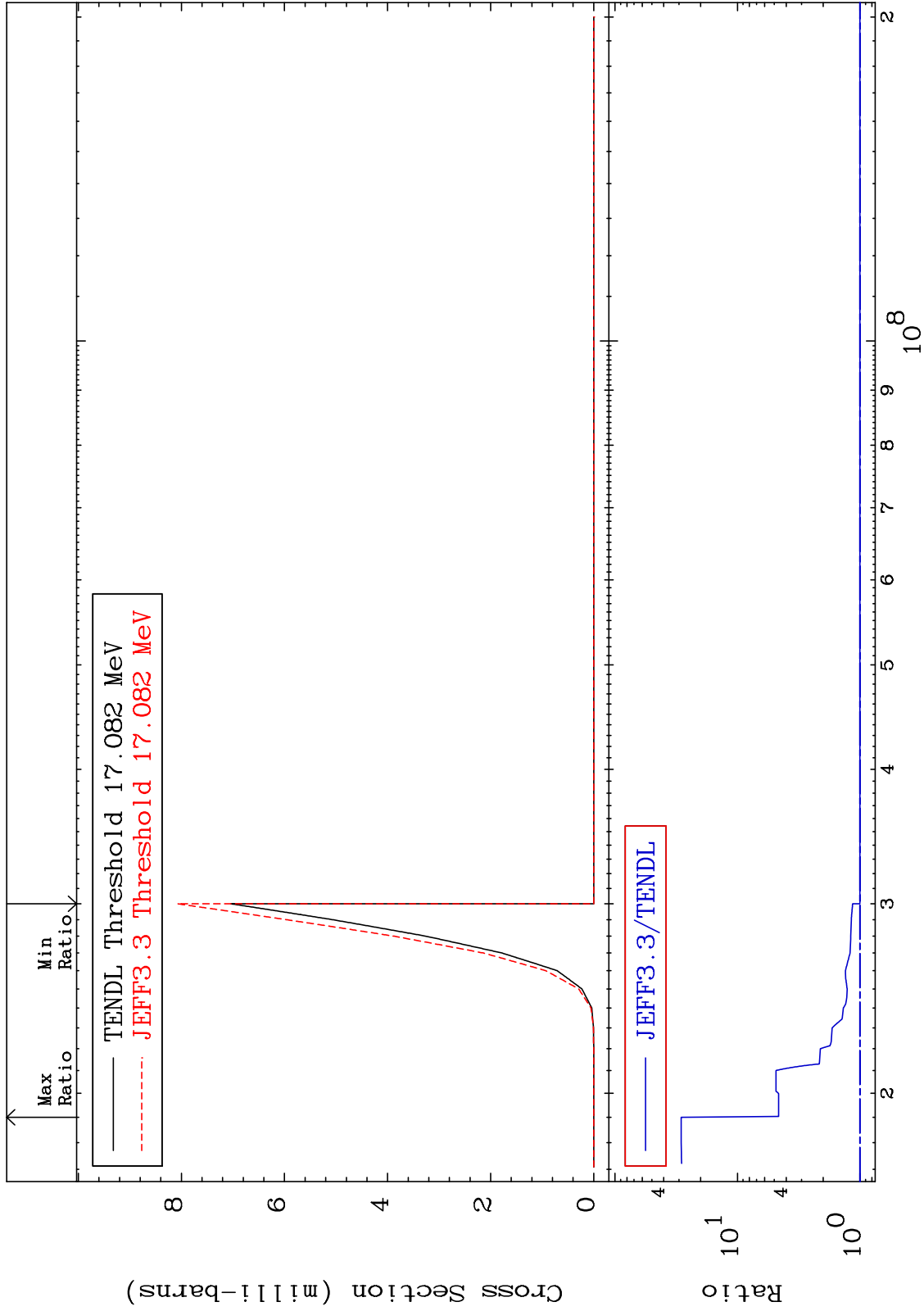
(n,2n) α

²⁶Fe-60

Cross Section

0.000

To 2780. %



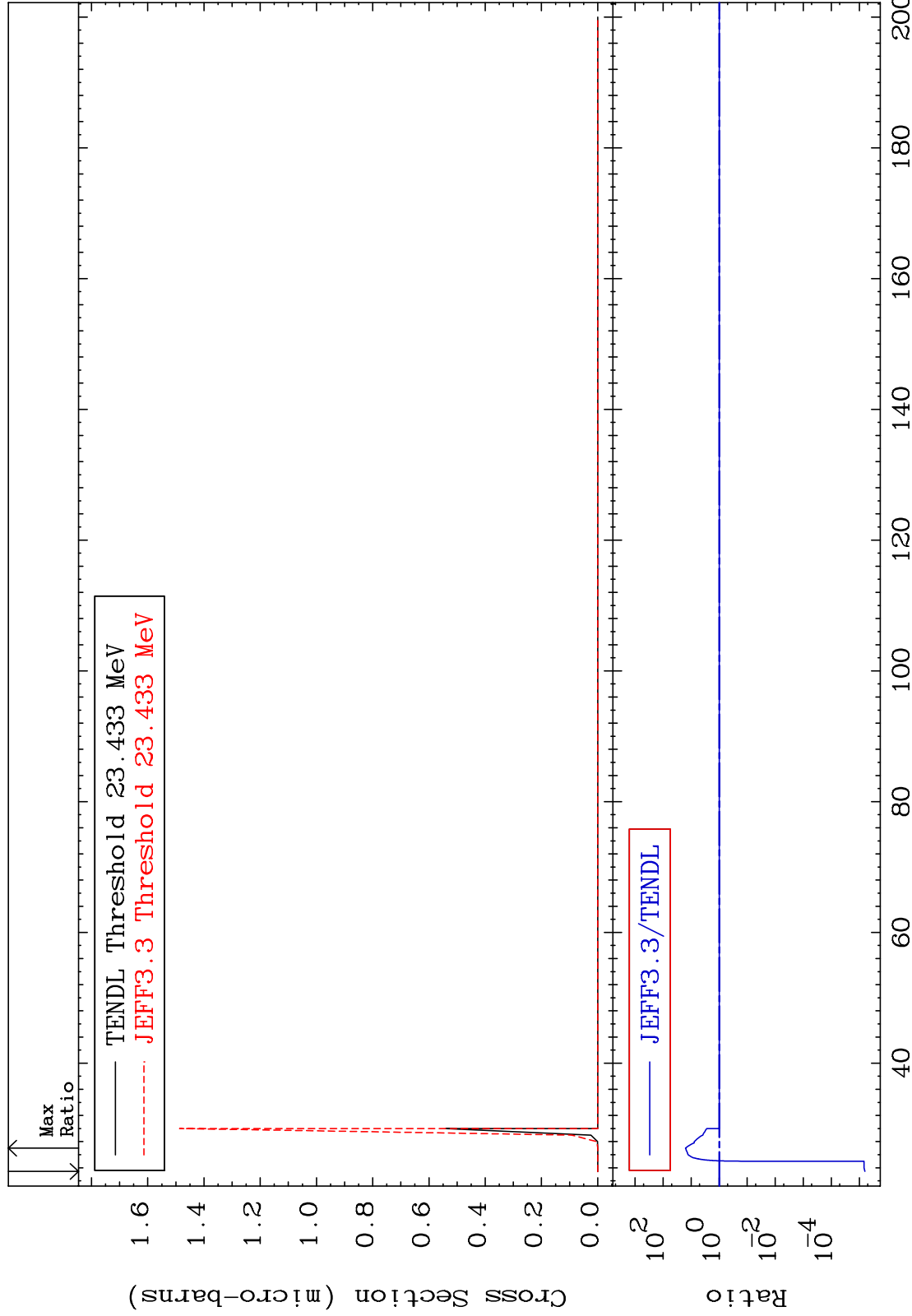
MAT 2643

(n, 3n) α

²⁶Fe-60

Cross Section

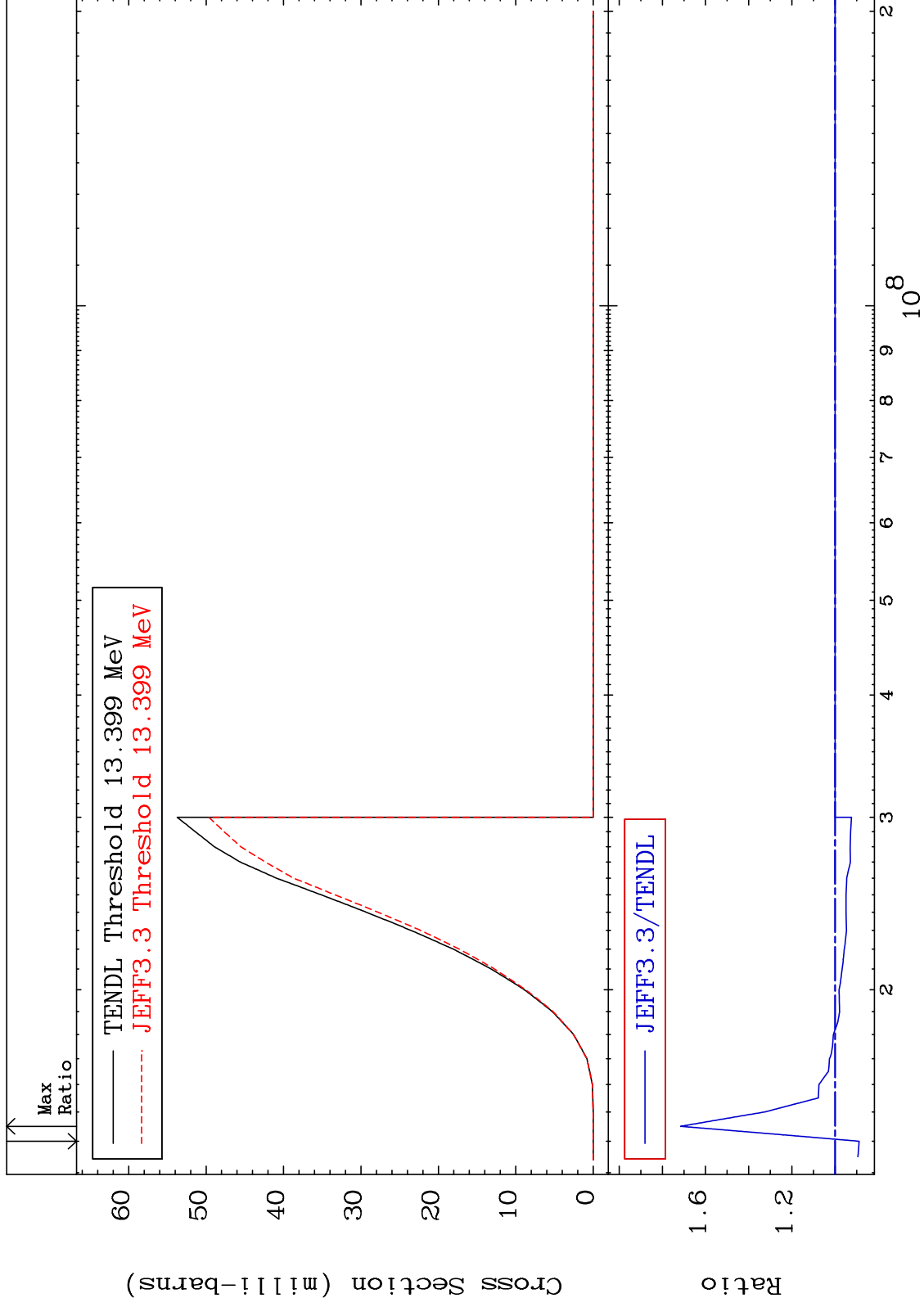
-100.0 To 1484. %



MAT 2643

(n, n') p
Cross Section

²⁶Fe-60
-11.18 To 71.55 %

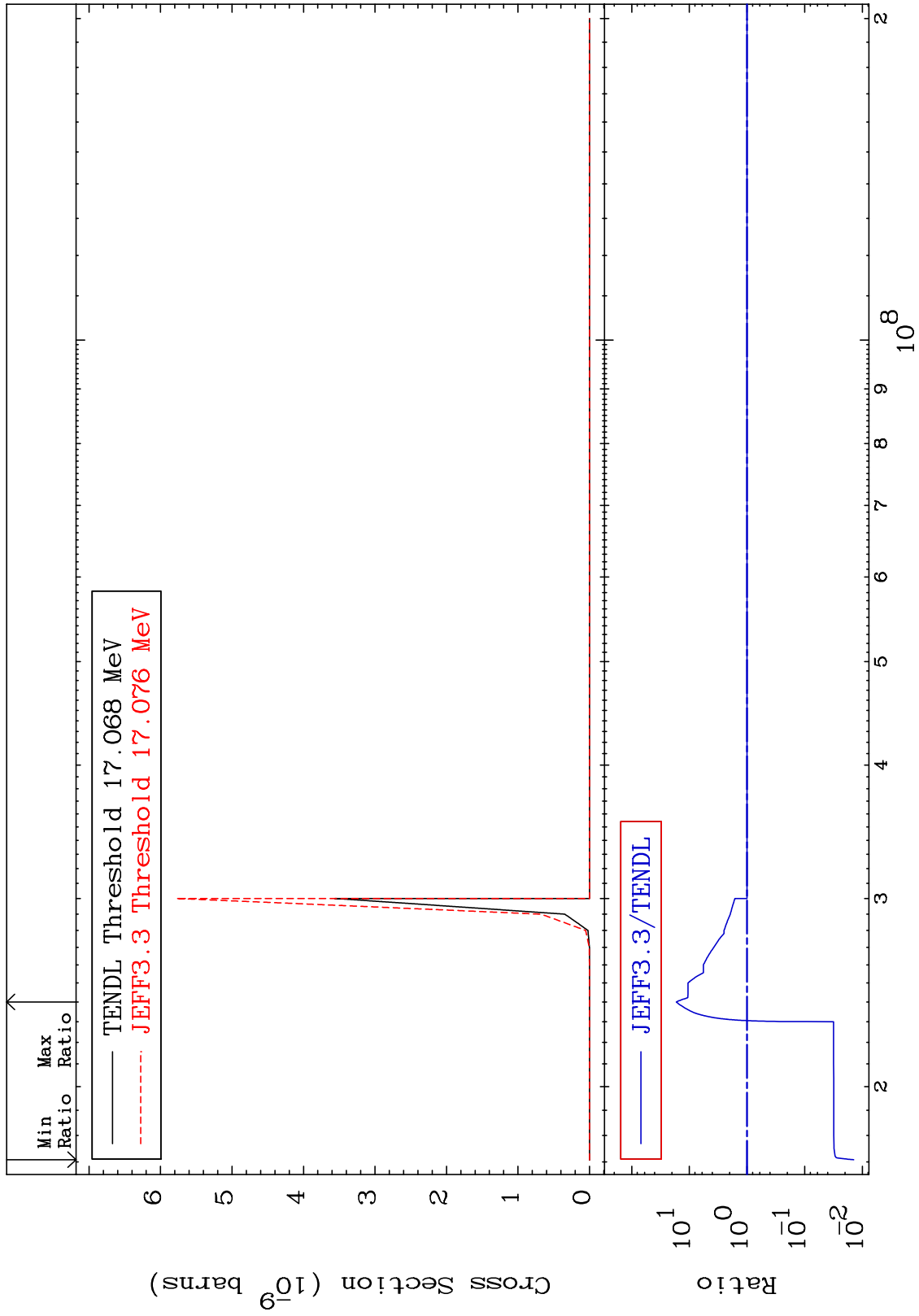


10

Incident Energy (eV)

²⁶Fe-60

MAT 2643 (n,n') 2α Cross Section 26-Fe-60 -98.59 To 1583. %



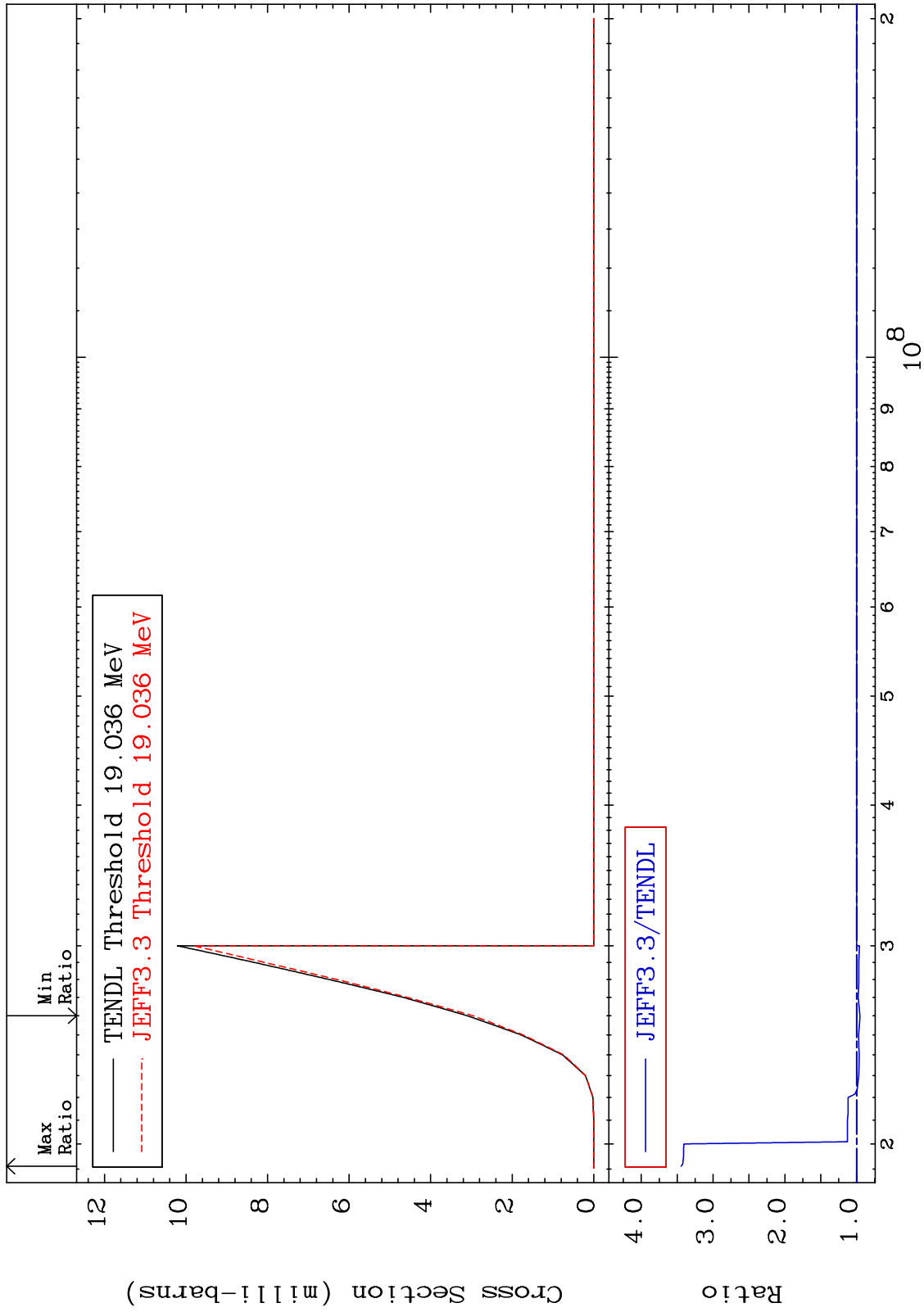
MAT 2643

(n,n') d

²⁶Fe-60

Cross Section

-4.509 To 244.4 %



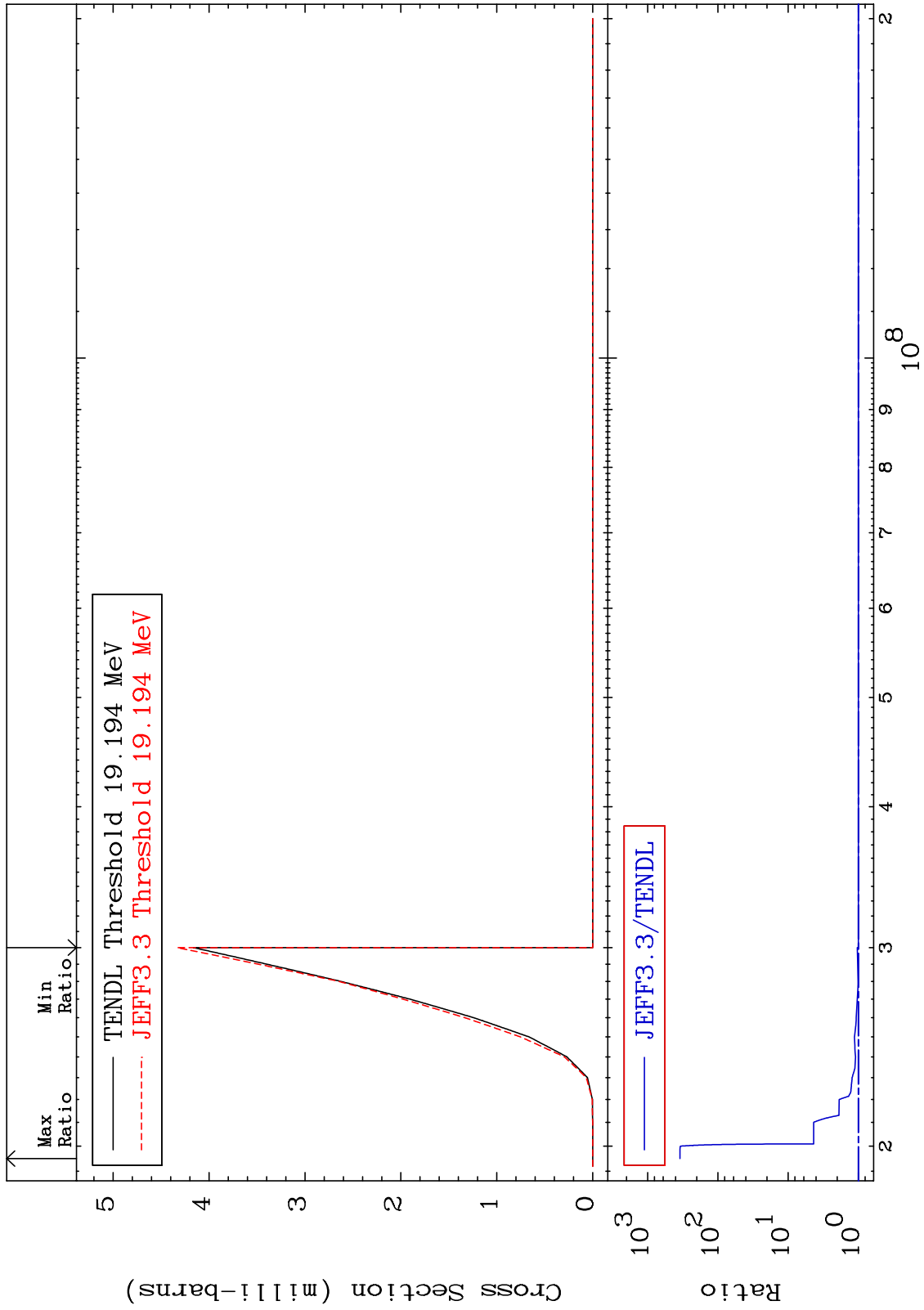
MAT 2643

(n,n') t

²⁶Fe-60

Cross Section

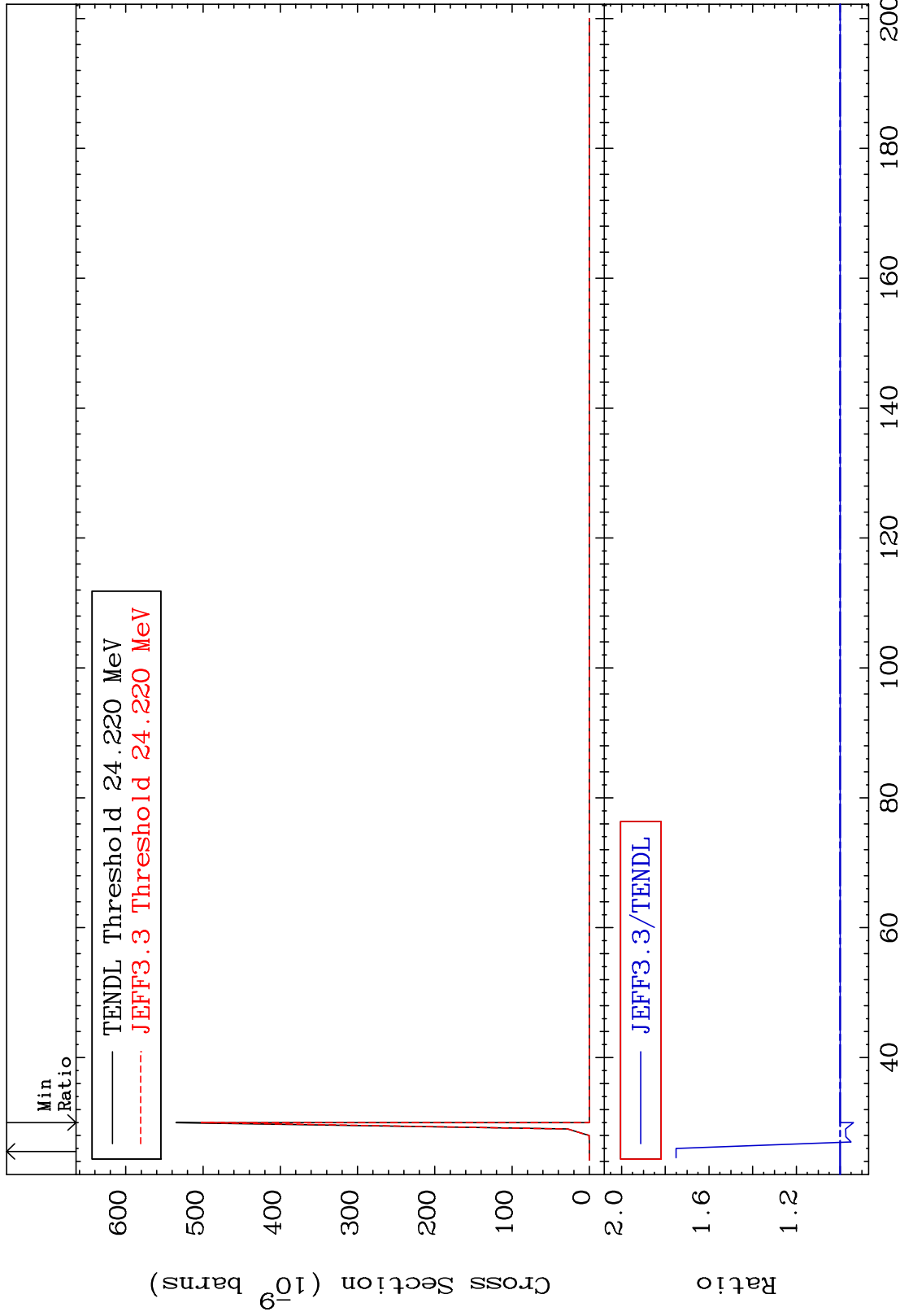
0.000 To 9999. %



MAT 2643

(n, n') He-3
Cross Section

26-Fe-60
-6.119 To 75.04 %



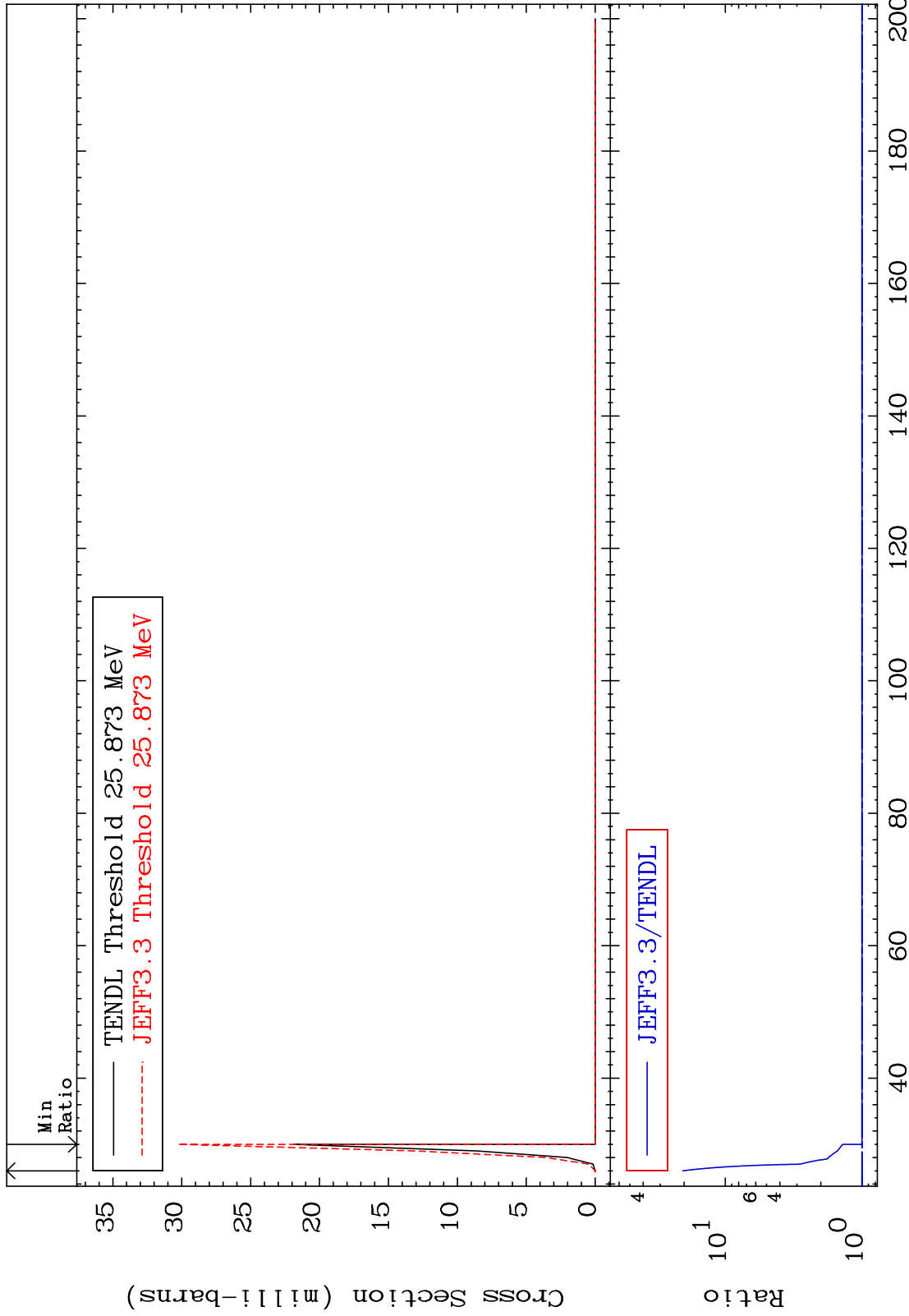
MAT 2643

(n, 4n)

²⁶Fe-60

Cross Section

0.000 To 1946. %



²⁶Fe-60

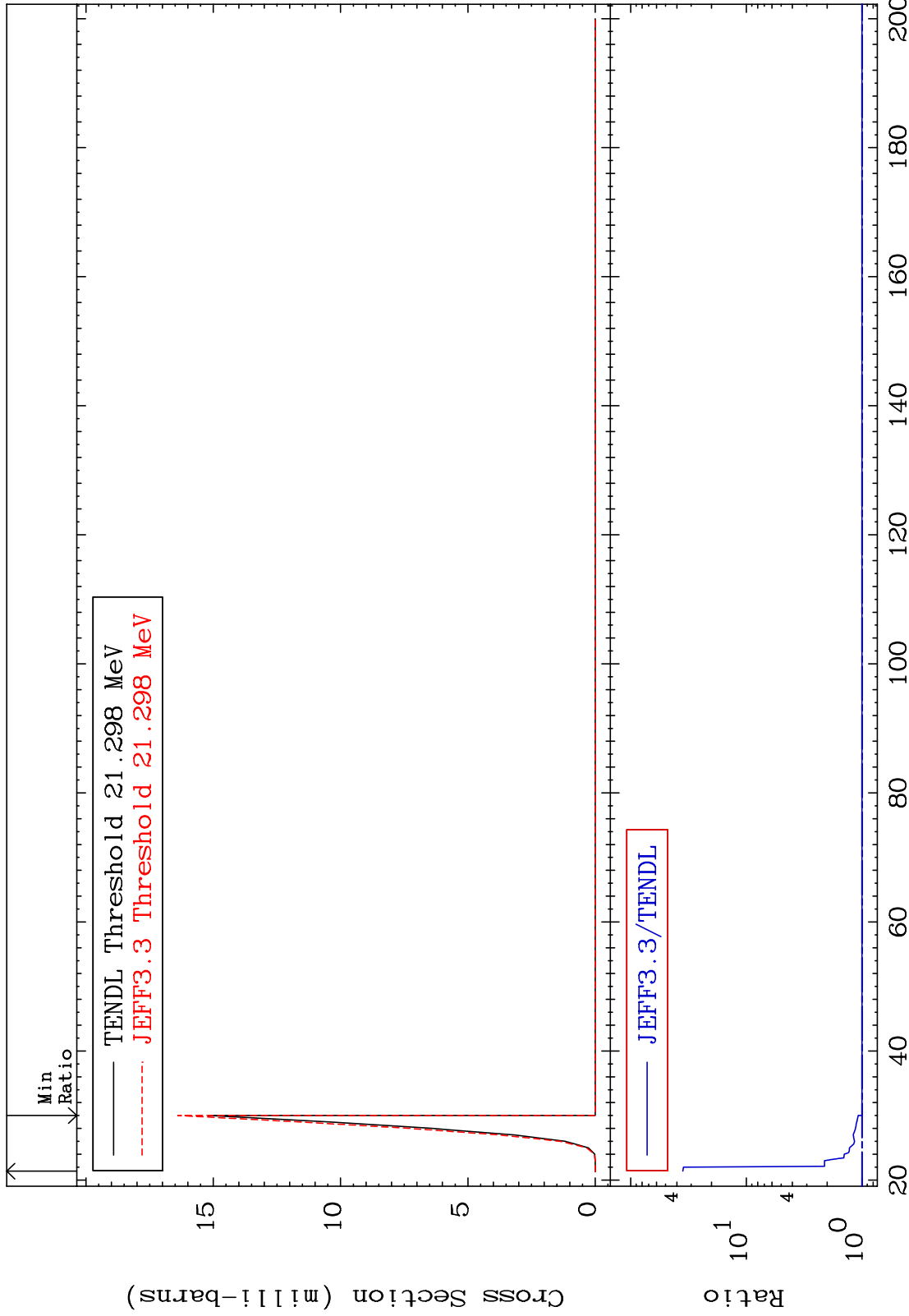
Incident Energy (MeV)

15

MAT 2643

(n,2n) p
Cross Section

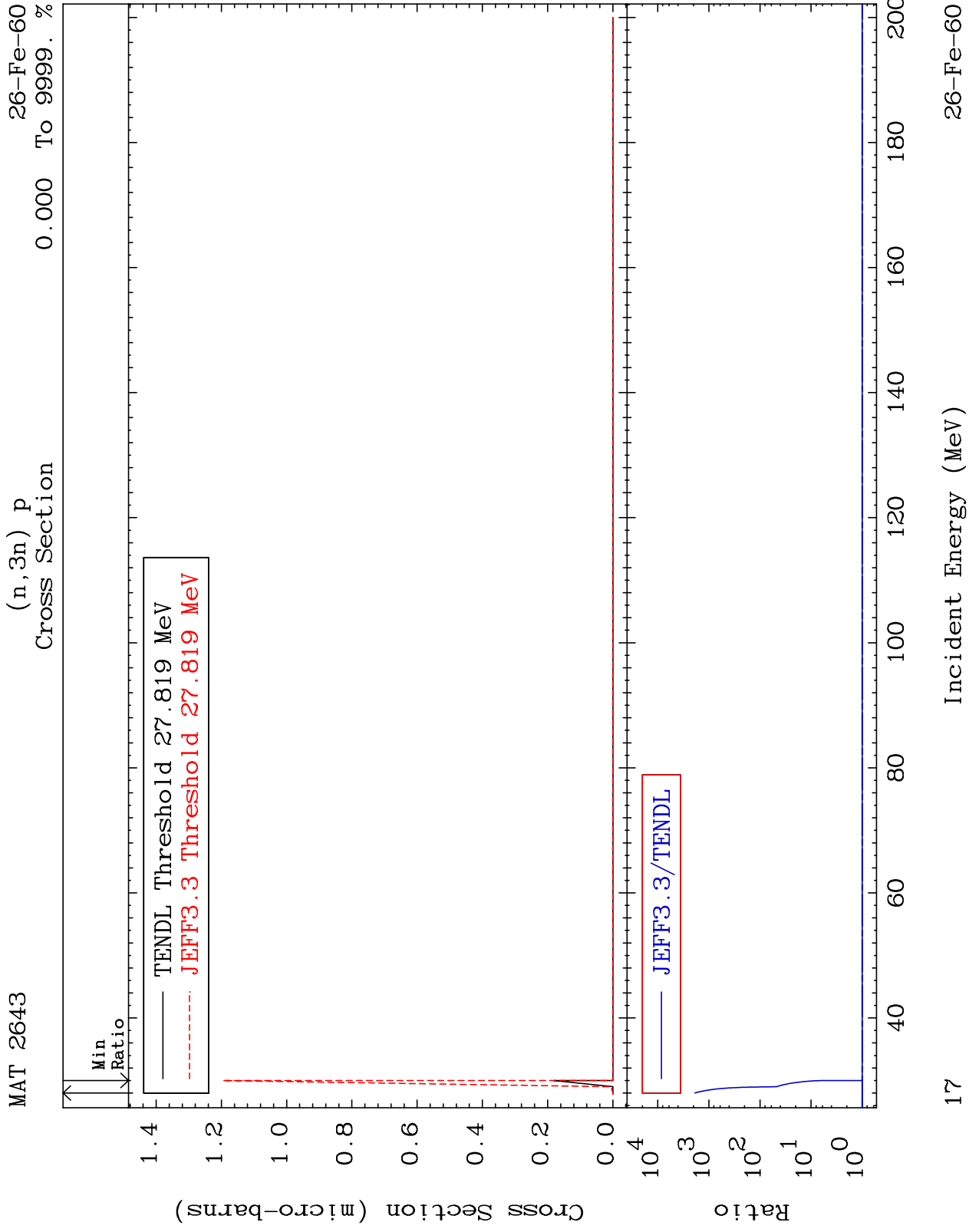
²⁶Fe-60
0.000 To 3444. %



16

Incident Energy (MeV)

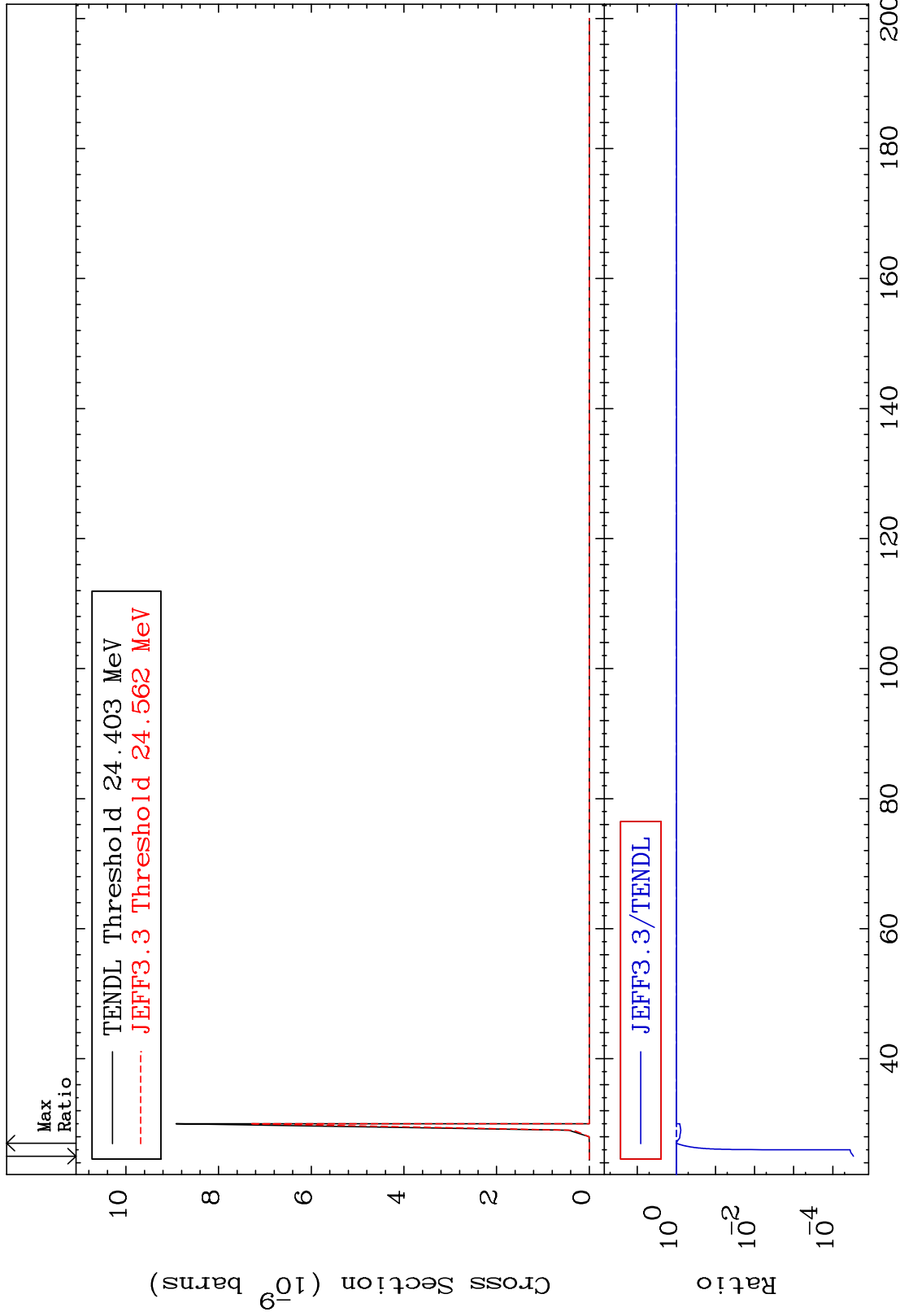
²⁶Fe-60



MAT 2643

(n,2n) p
Cross Section

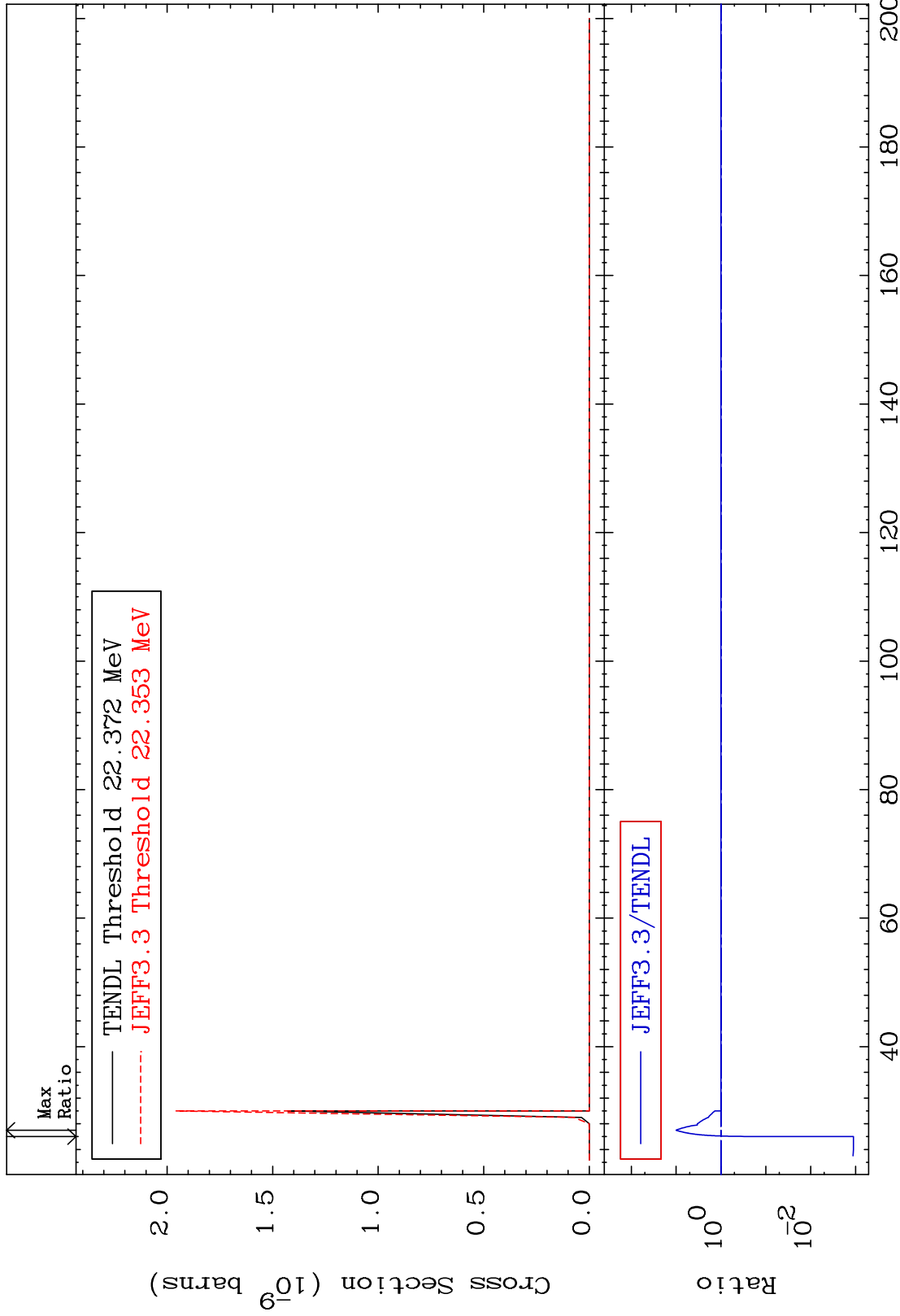
26-Fe-60
-100.0 To 1.788 %



MAT 2643

(n,n') p α
Cross Section

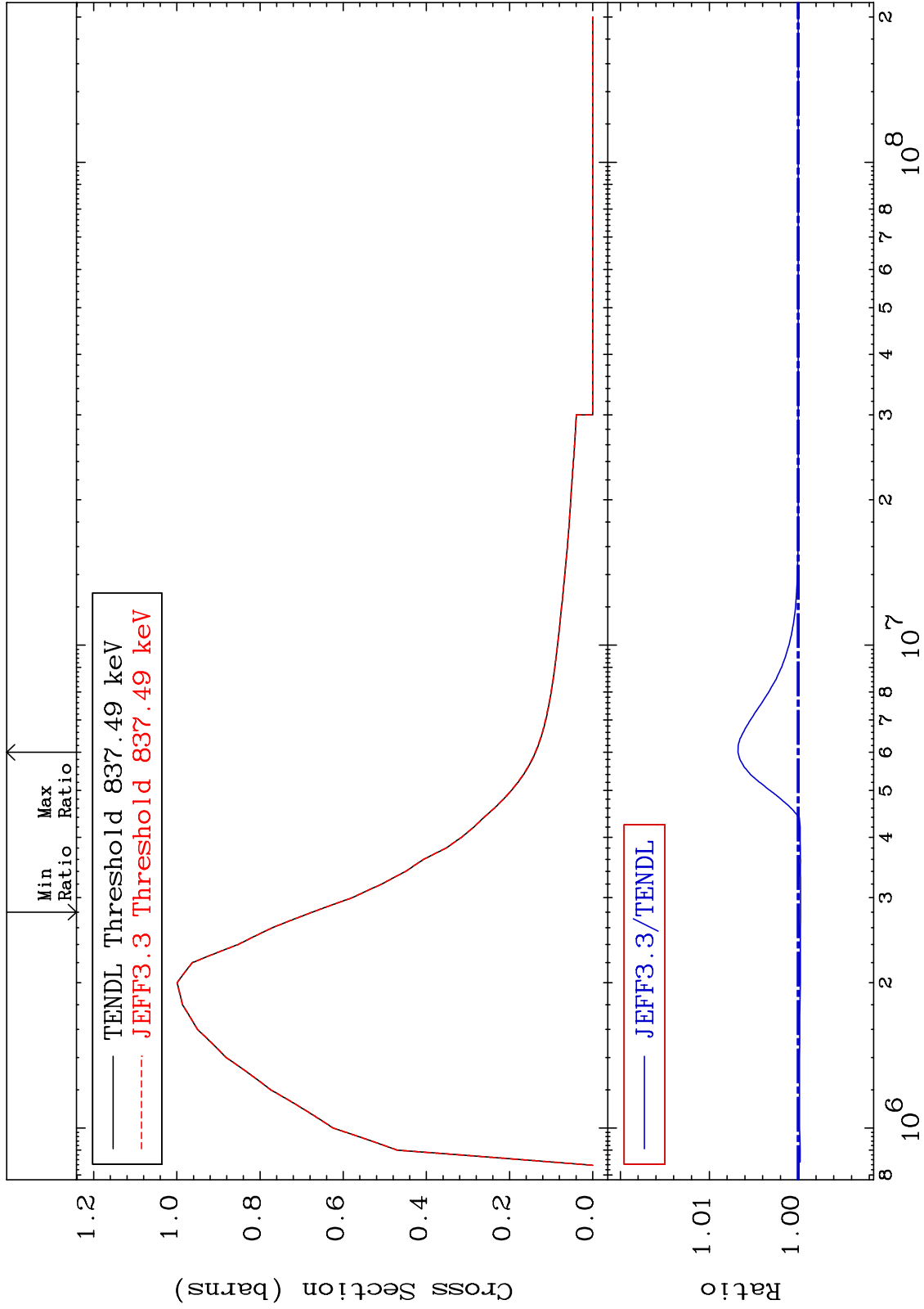
26-Fe-60
-99.89 To 900.3 %



MAT 2643

MT= 51 (n,n') Level
Cross Section

26-Fe-60
-0.026 To 0.678 %

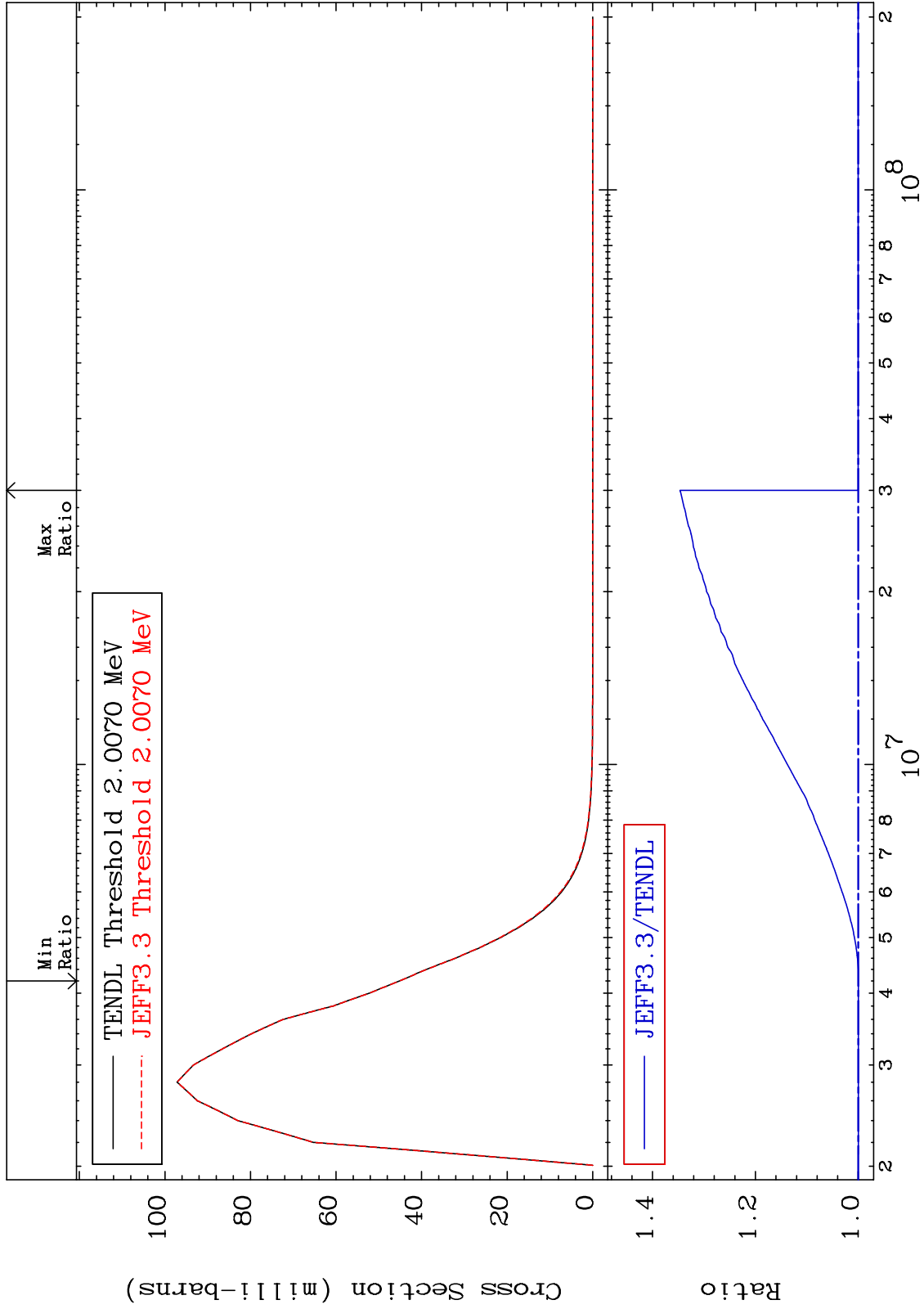


20 26-Fe-60

MAT 2643

MT= 52 (n,n') Level
Cross Section

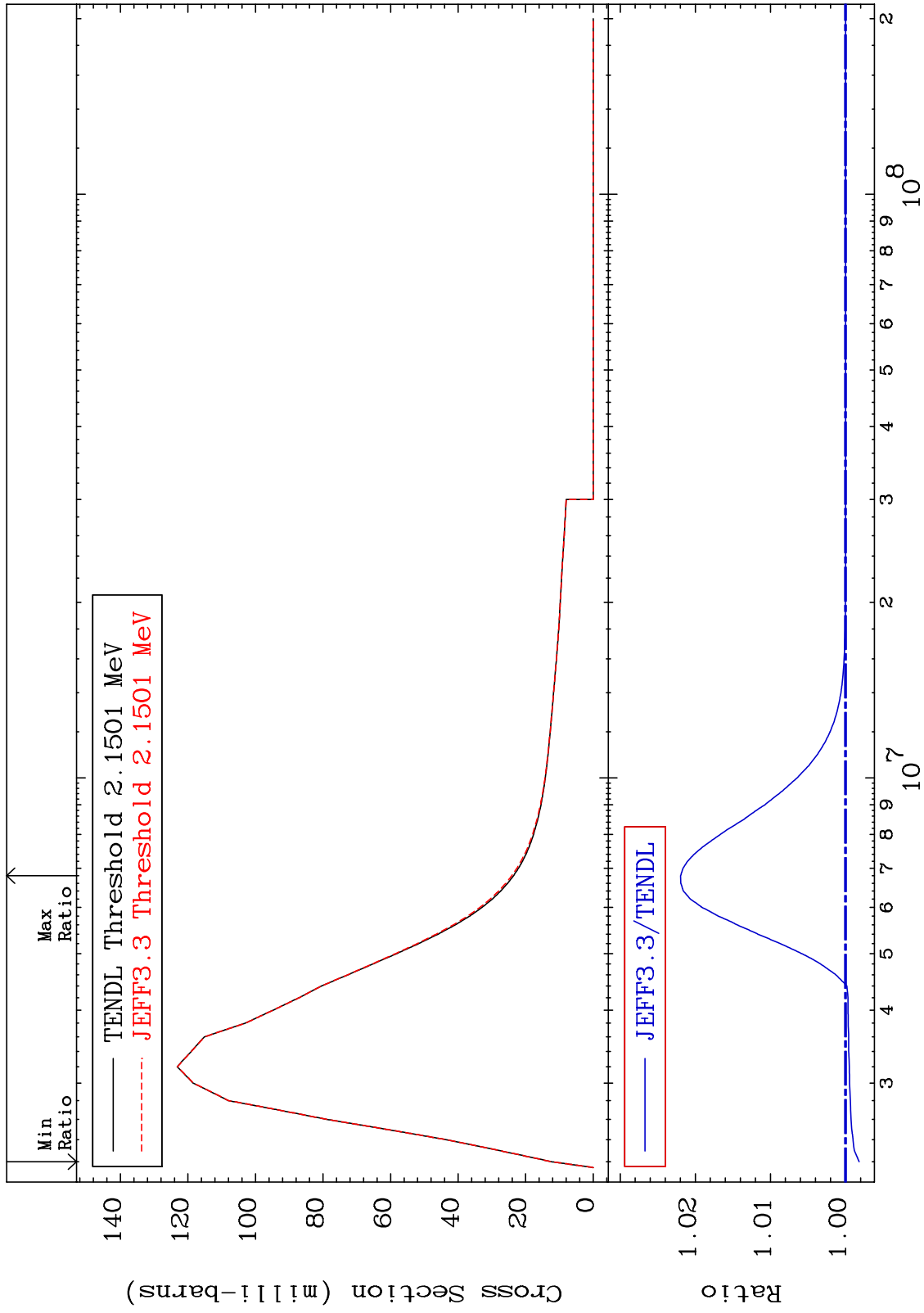
26-Fe-60
-0.033 To 34.66 %



MAT 2643

MT= 53 (n,n') Level
Cross Section

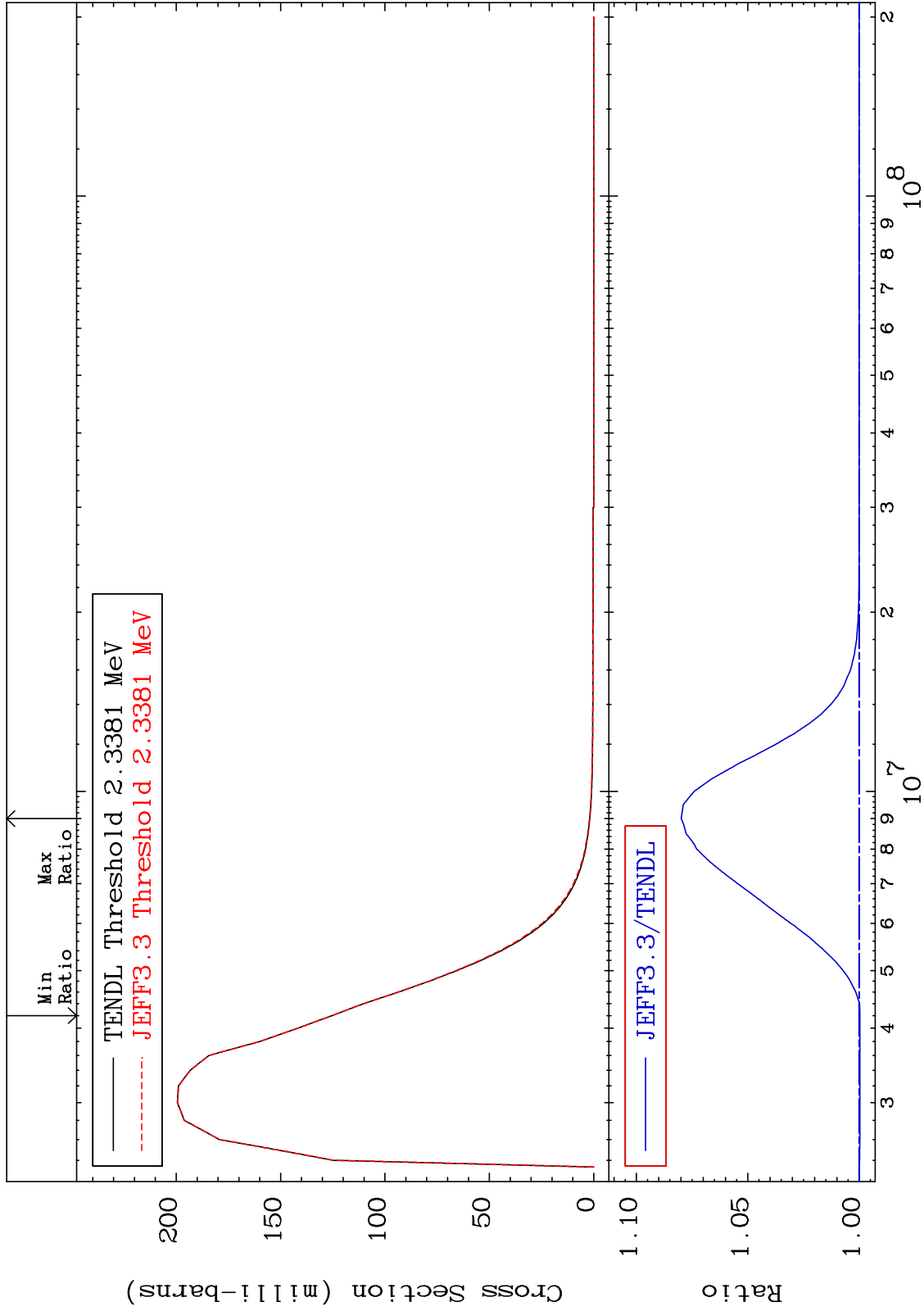
26-Fe-60
-0.182 To 2.196 %



MAT 2643

MT= 54 (n, n') Level
Cross Section

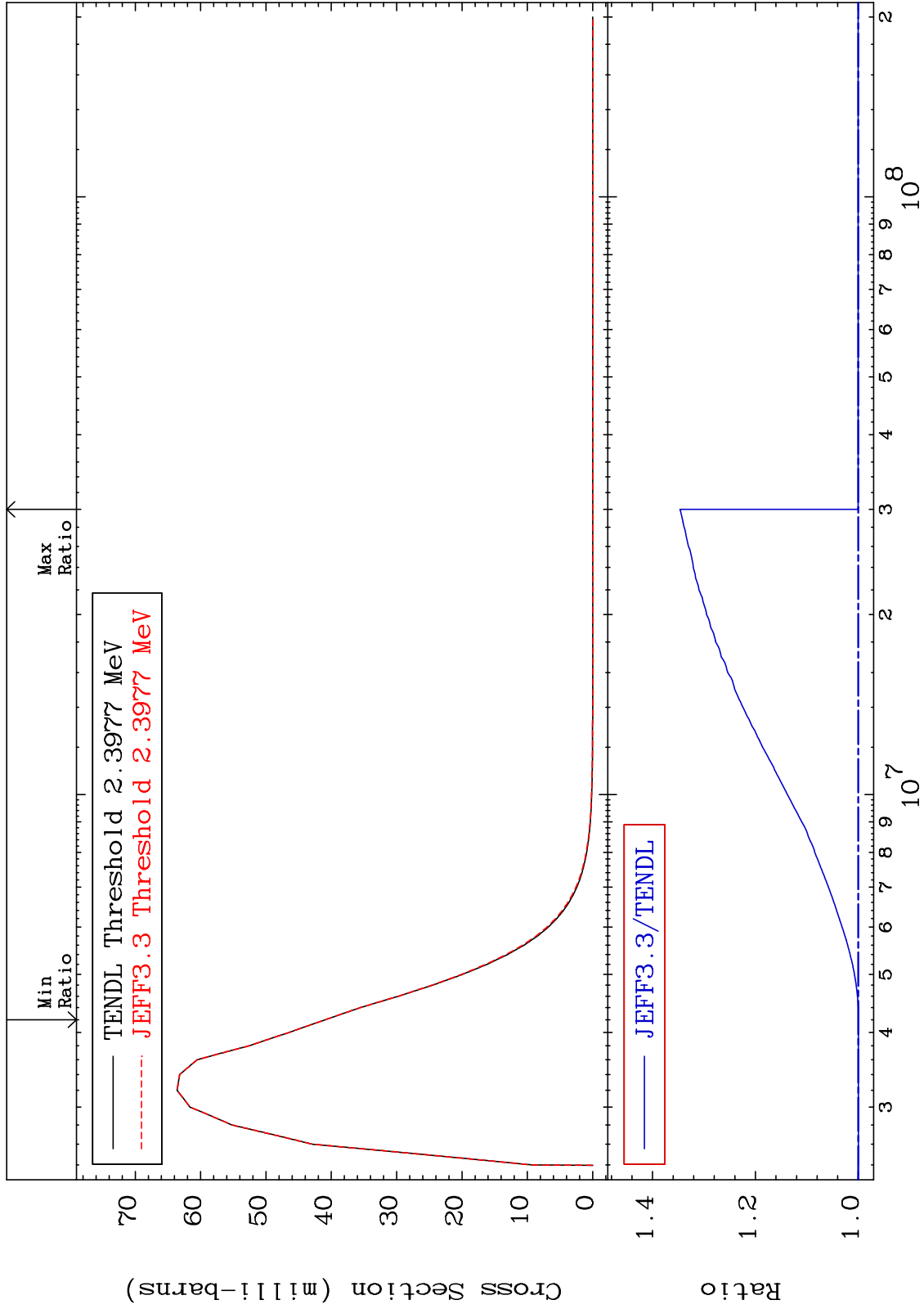
26-Fe-60
-0.033 To 7.985 %



MAT 2643

MT= 55 (n, n') Level
Cross Section

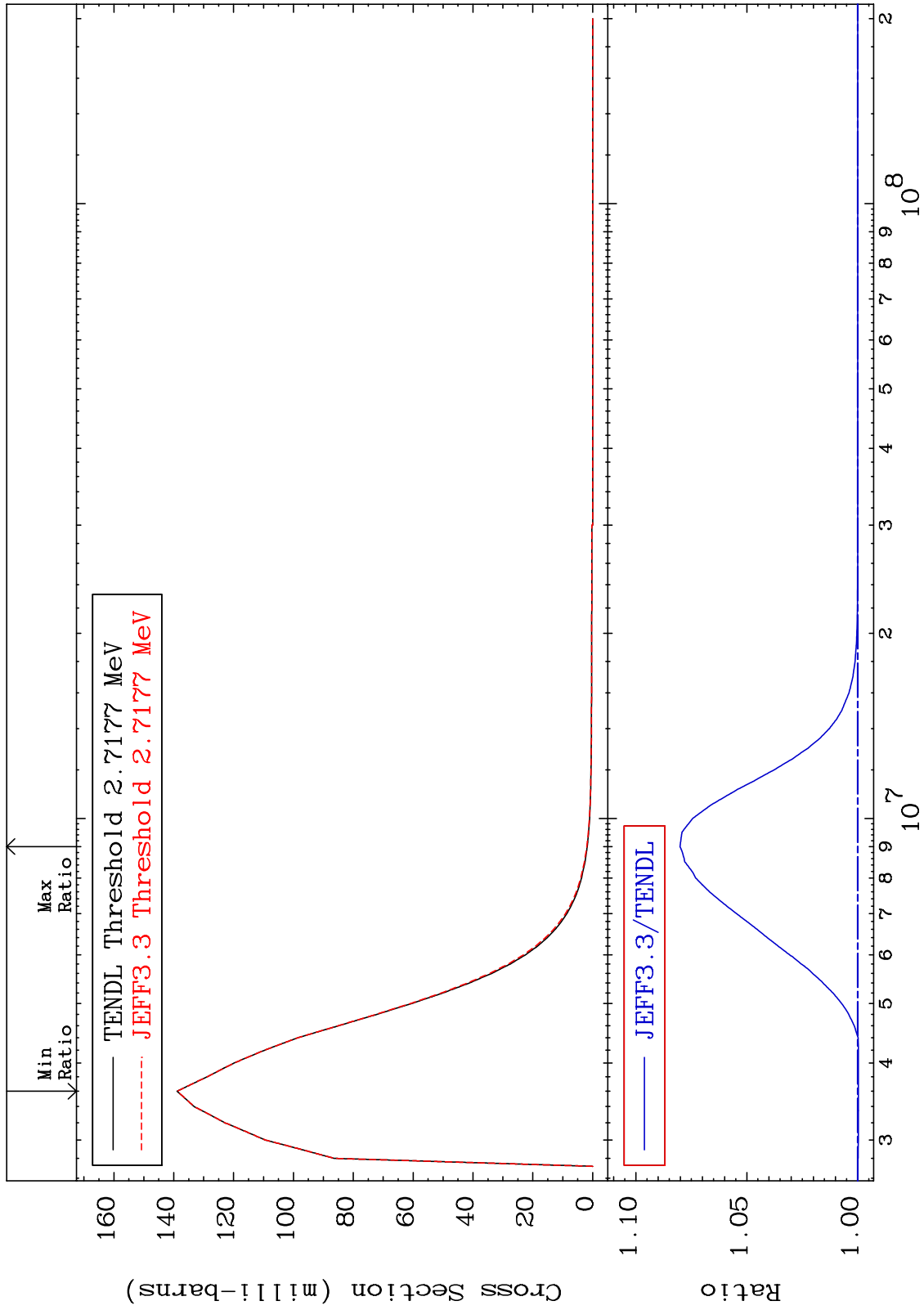
26-Fe-60
-0.032 To 34.66 %



MAT 2643

MT= 56 (n, n') Level
Cross Section

26-Fe-60
-0.032 To 8.012 %



25

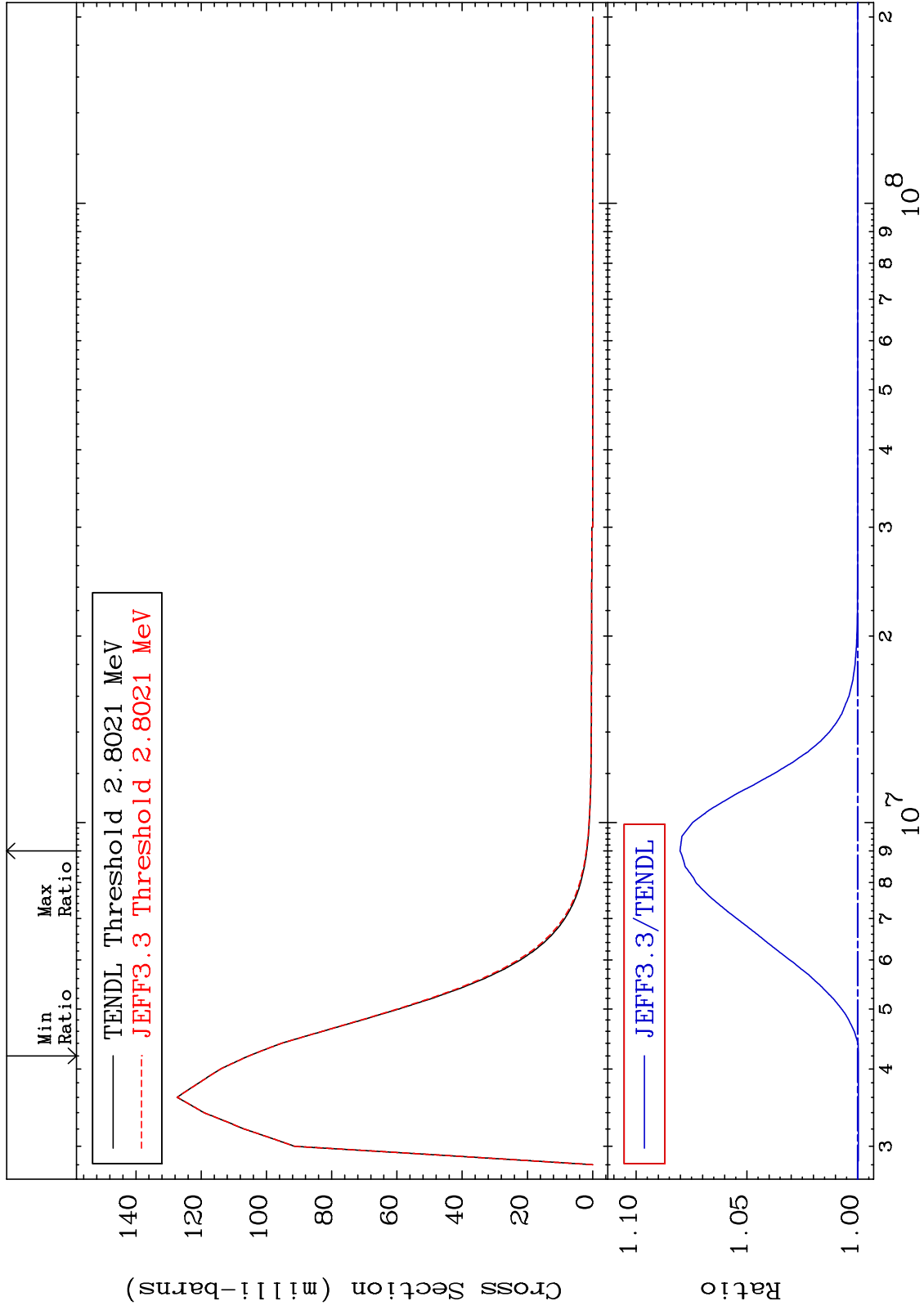
Incident Energy (eV)

26-Fe-60

MAT 2643

MT= 57 (n, n') Level
Cross Section

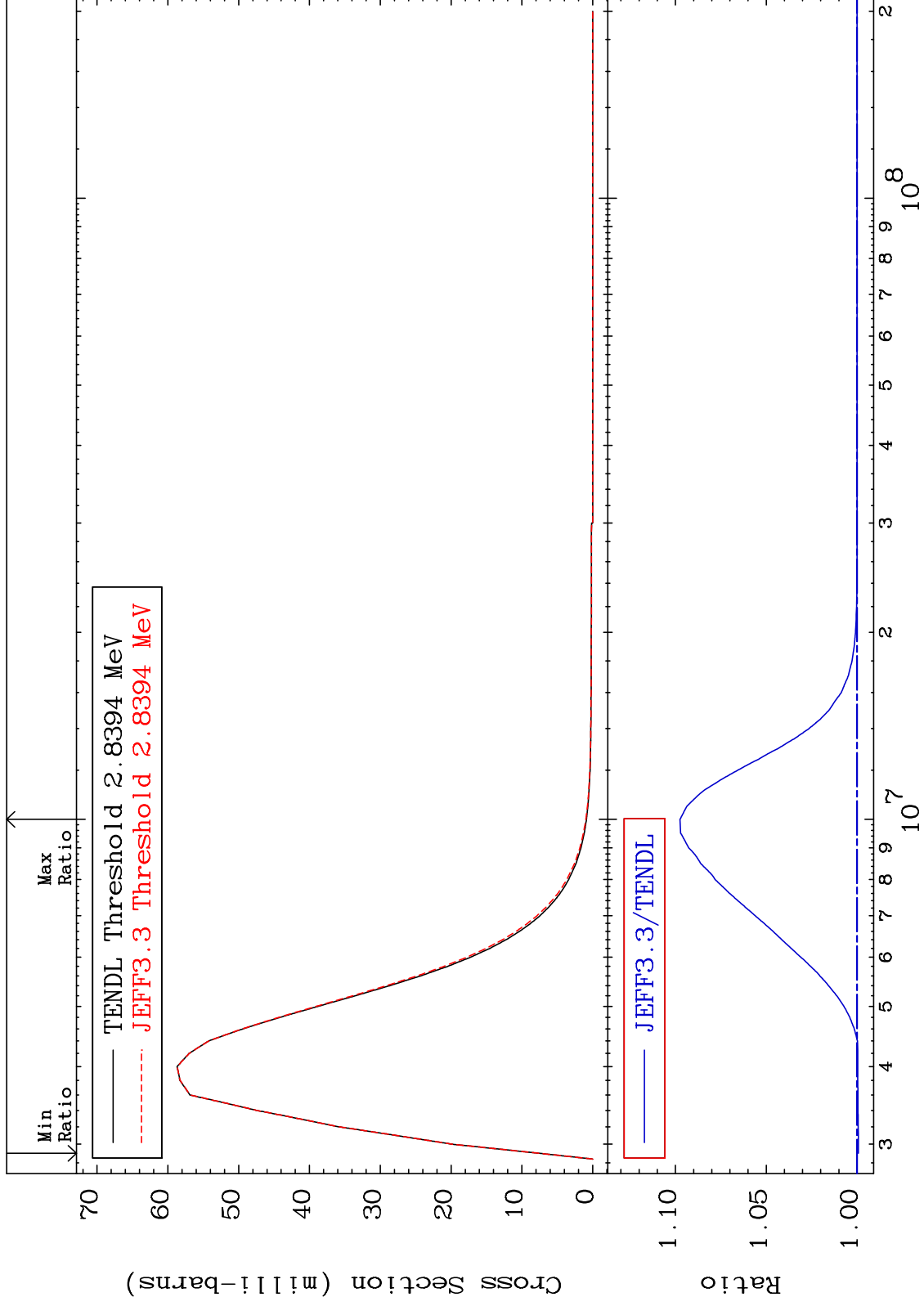
26-Fe-60
-0.032 To 8.019 %



MAT 2643

MT= 58 (n,n') Level
Cross Section

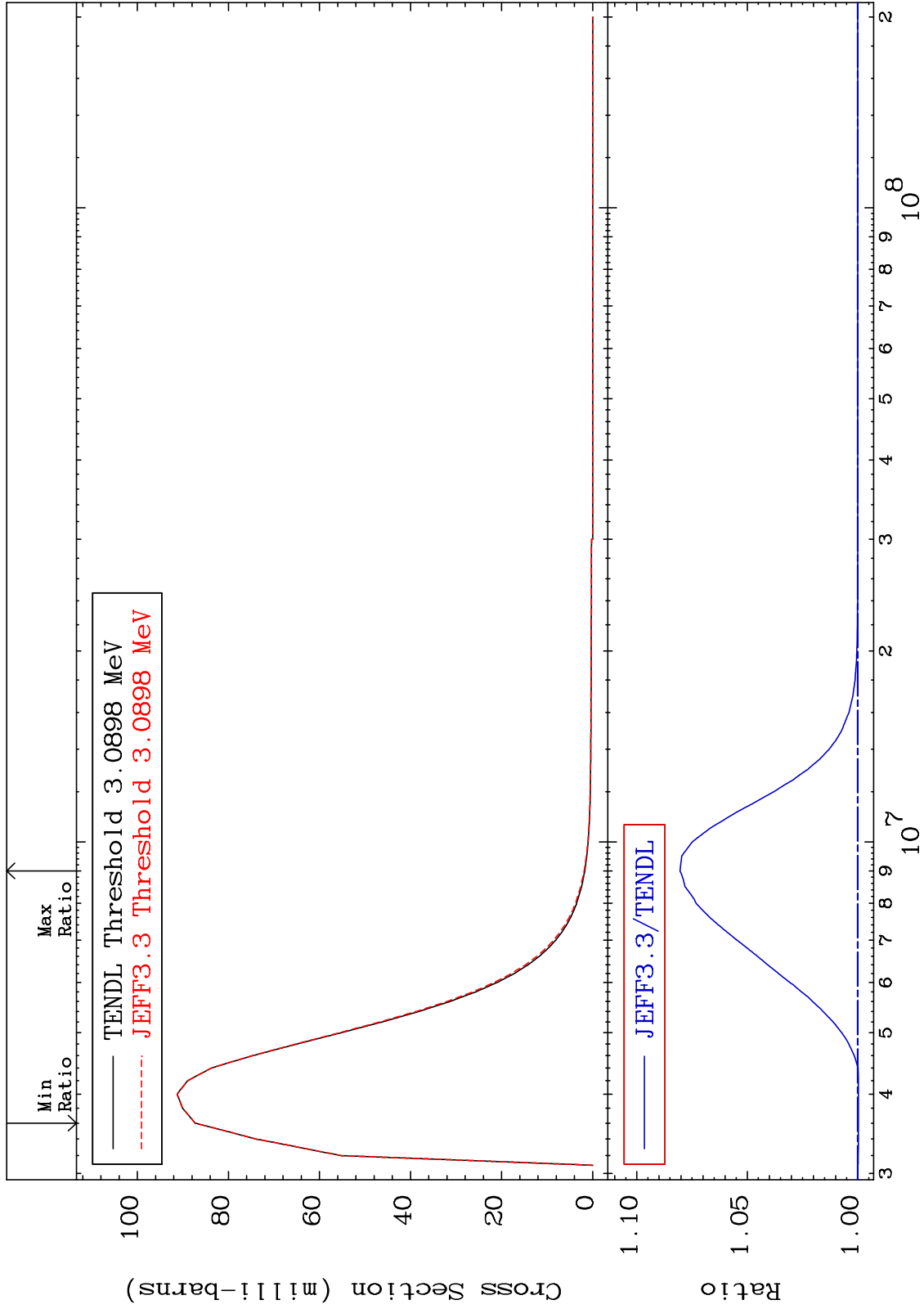
26-Fe-60
-0.076 To 9.747 %



MAT 2643

MT= 59 (n, n') Level
Cross Section

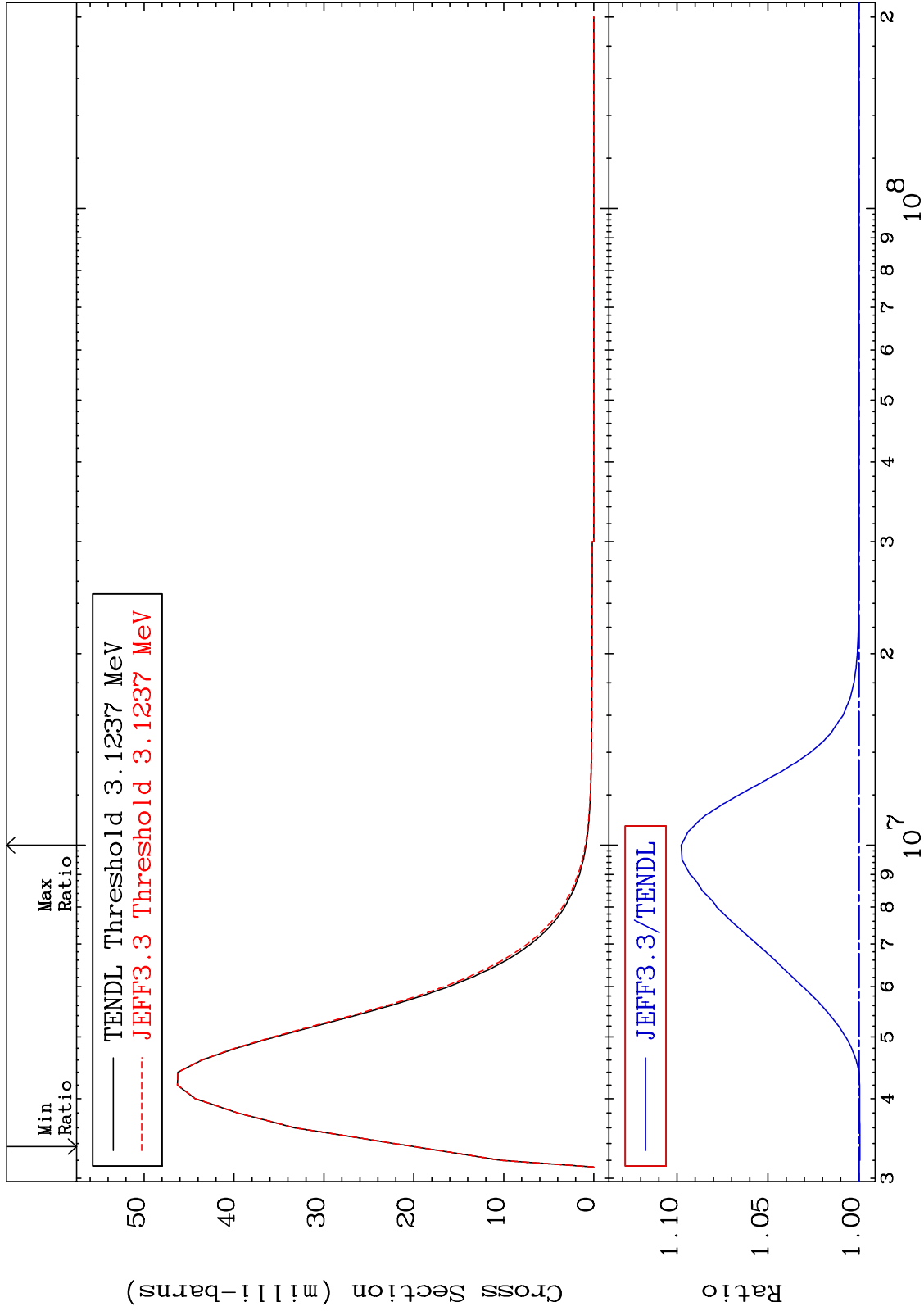
26-Fe-60
-0.032 To 8.045 %



MAT 2643

MT= 60 (n,n') Level
Cross Section

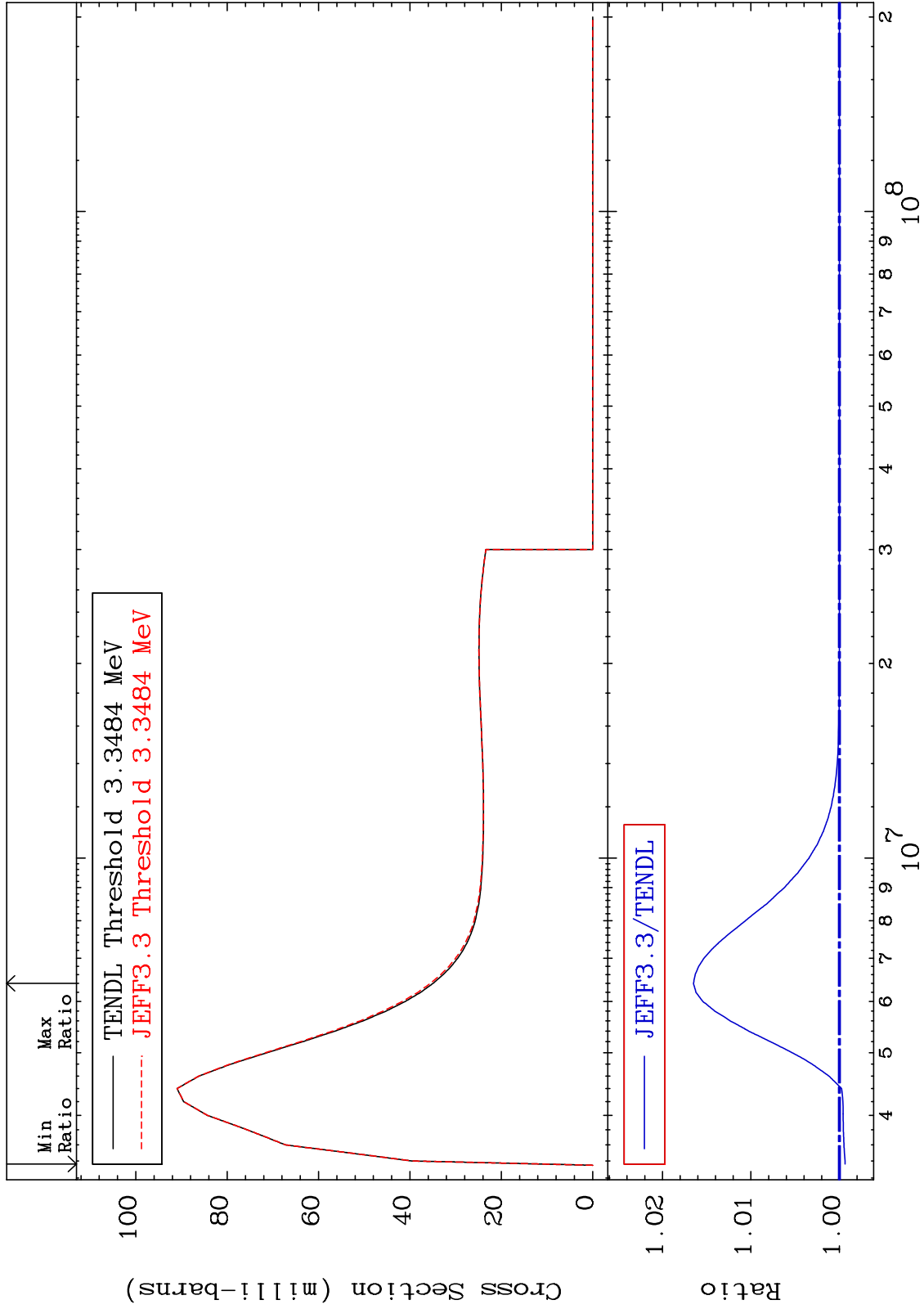
26-Fe-60
-0.056 To 9.771 %



MAT 2643

MT= 61 (n,n') Level
Cross Section

26-Fe-60
-0.063 To 1.649 %



30

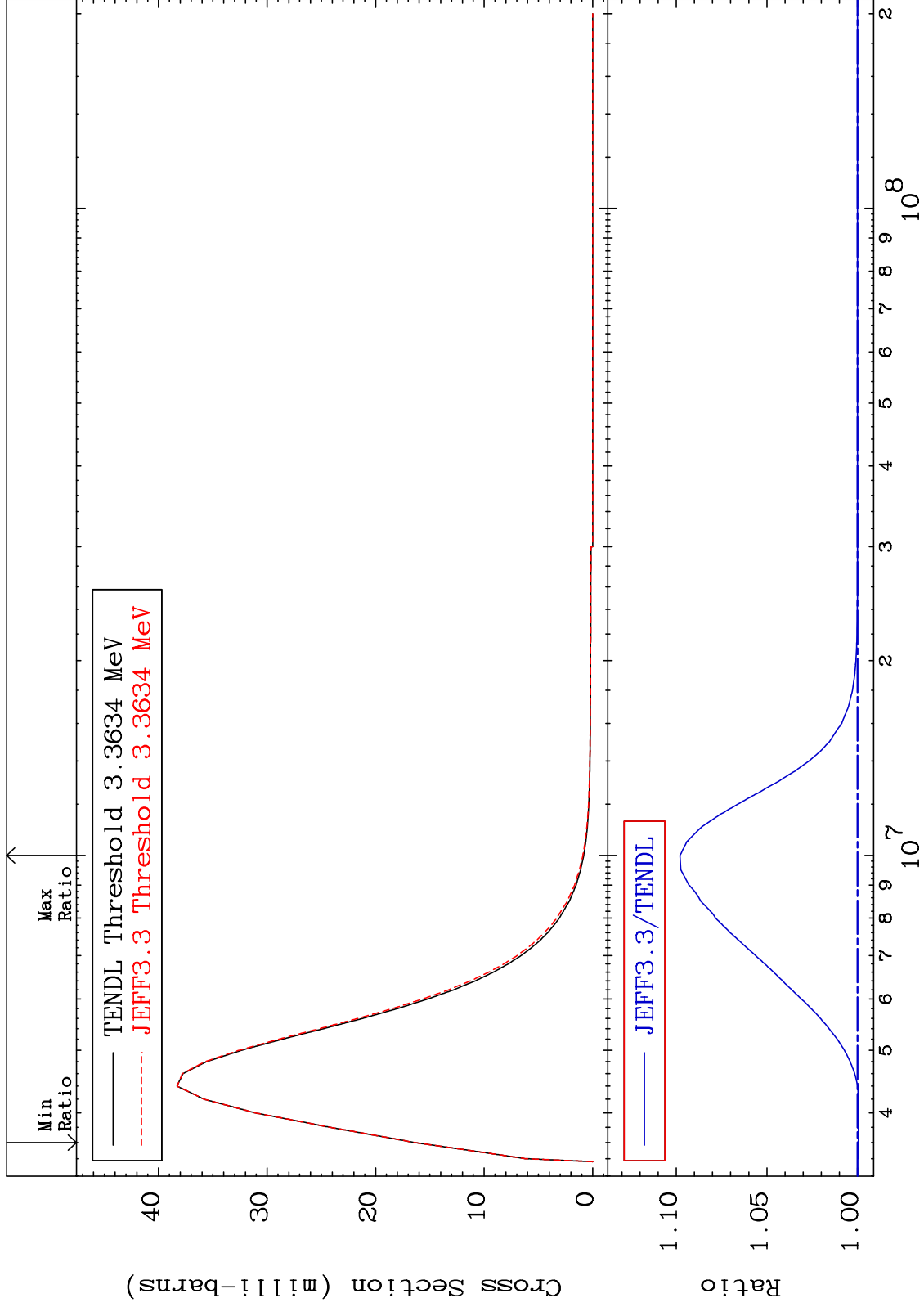
Incident Energy (eV)

26-Fe-60

MAT 2643

MT= 62 (n,n') Level
Cross Section

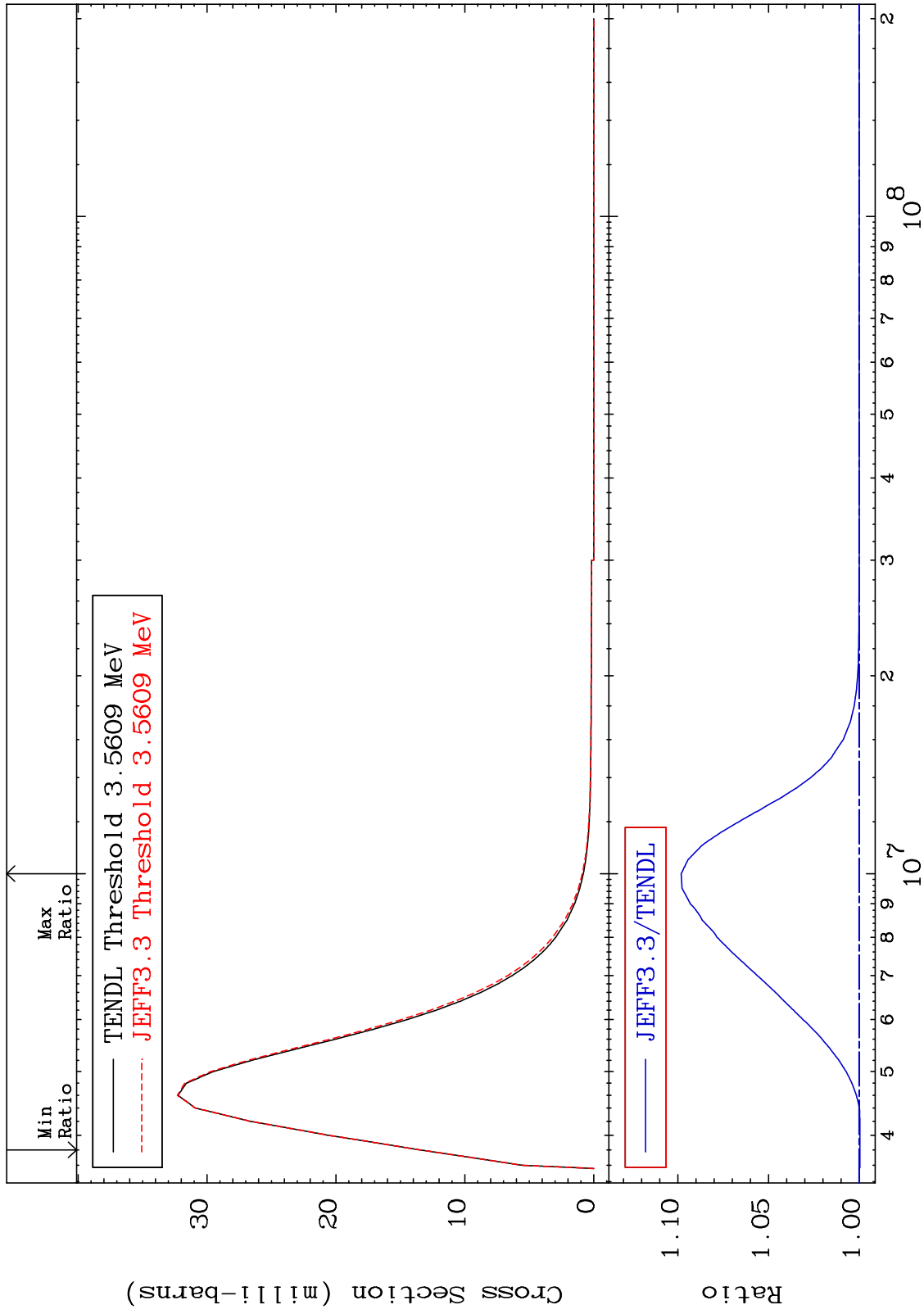
26-Fe-60
-0.051 To 9.795 %



MAT 2643

MT= 63 (n,n') Level
Cross Section

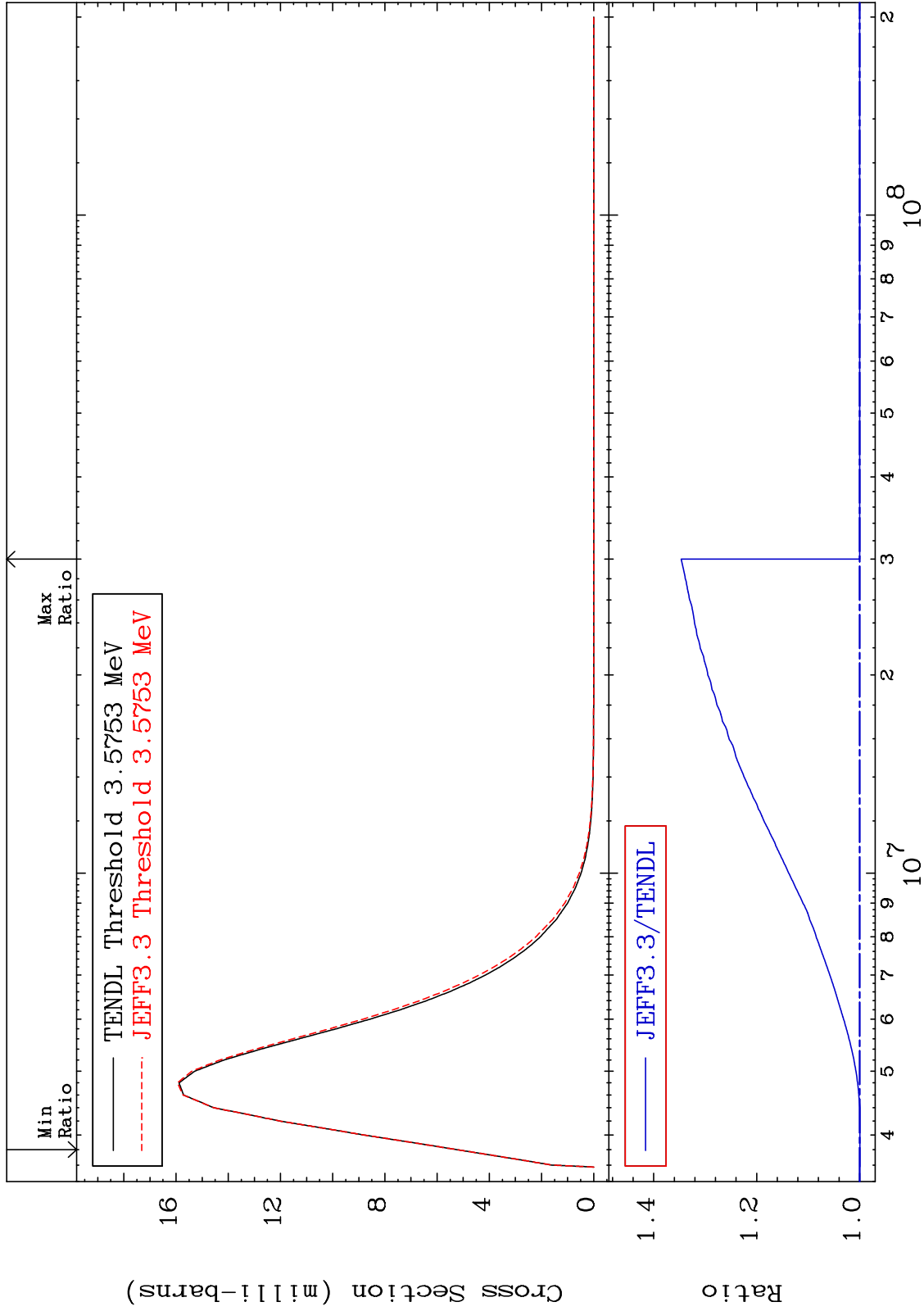
26-Fe-60
-0.041 To 9.818 %



MAT 2643

MT= 64 (n, n') Level
Cross Section

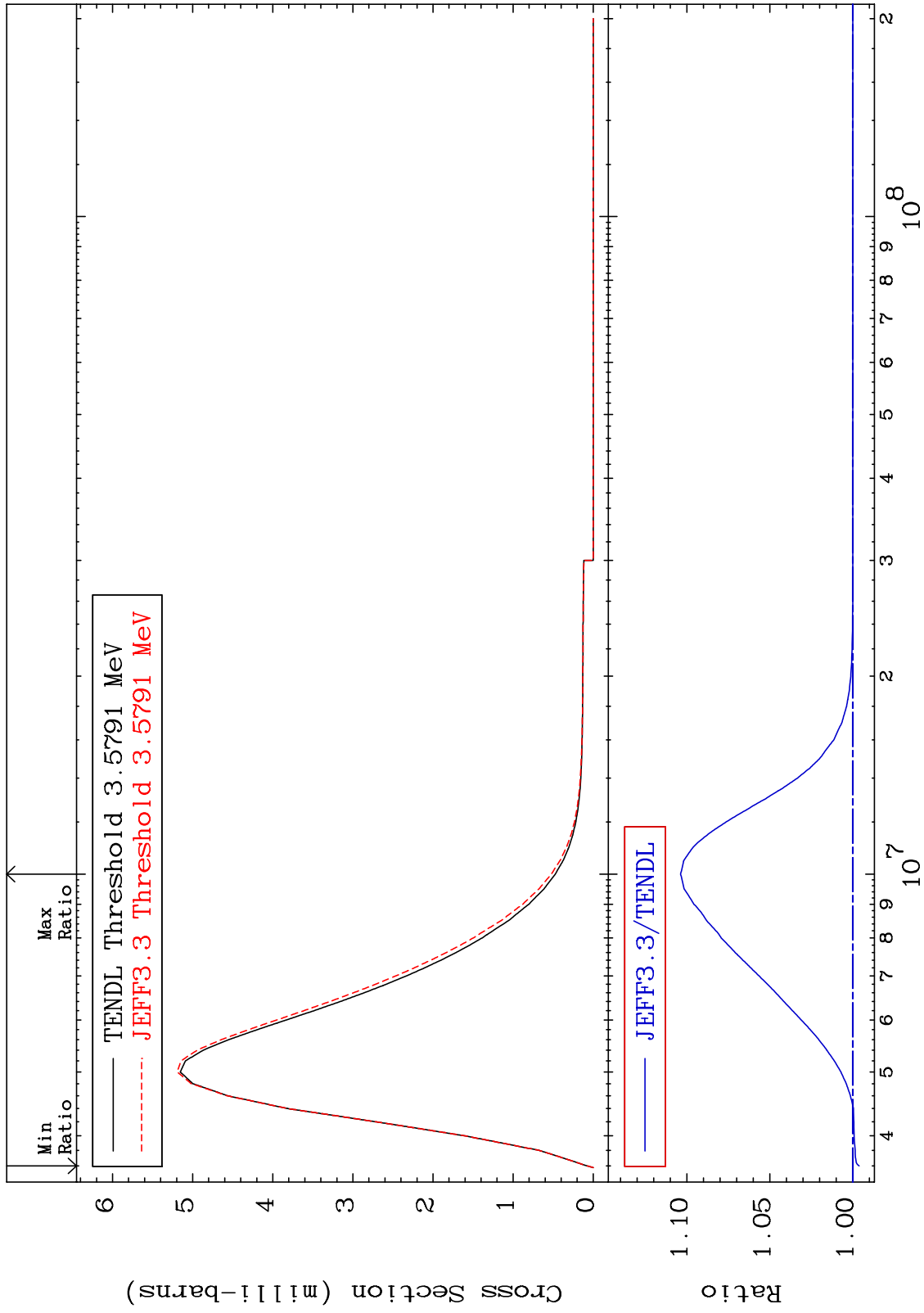
26-Fe-60
-0.053 To 34.66 %



MAT 2643

MT= 65 (n,n') Level
Cross Section

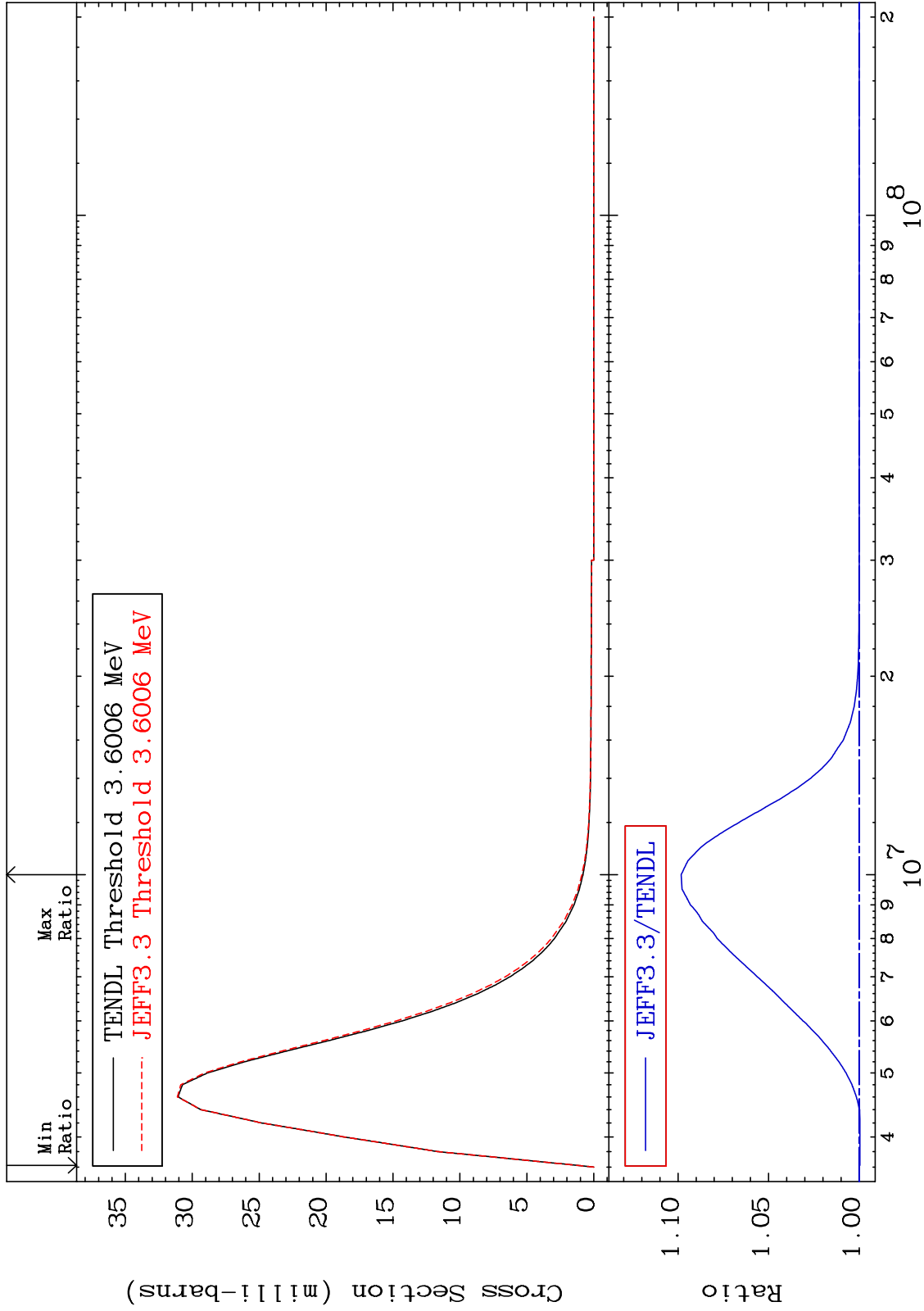
26-Fe-60
-0.385 To 10.38 %



MAT 2643

MT= 66 (n,n') Level
Cross Section

26-Fe-60
-0.041 To 9.823 %



35

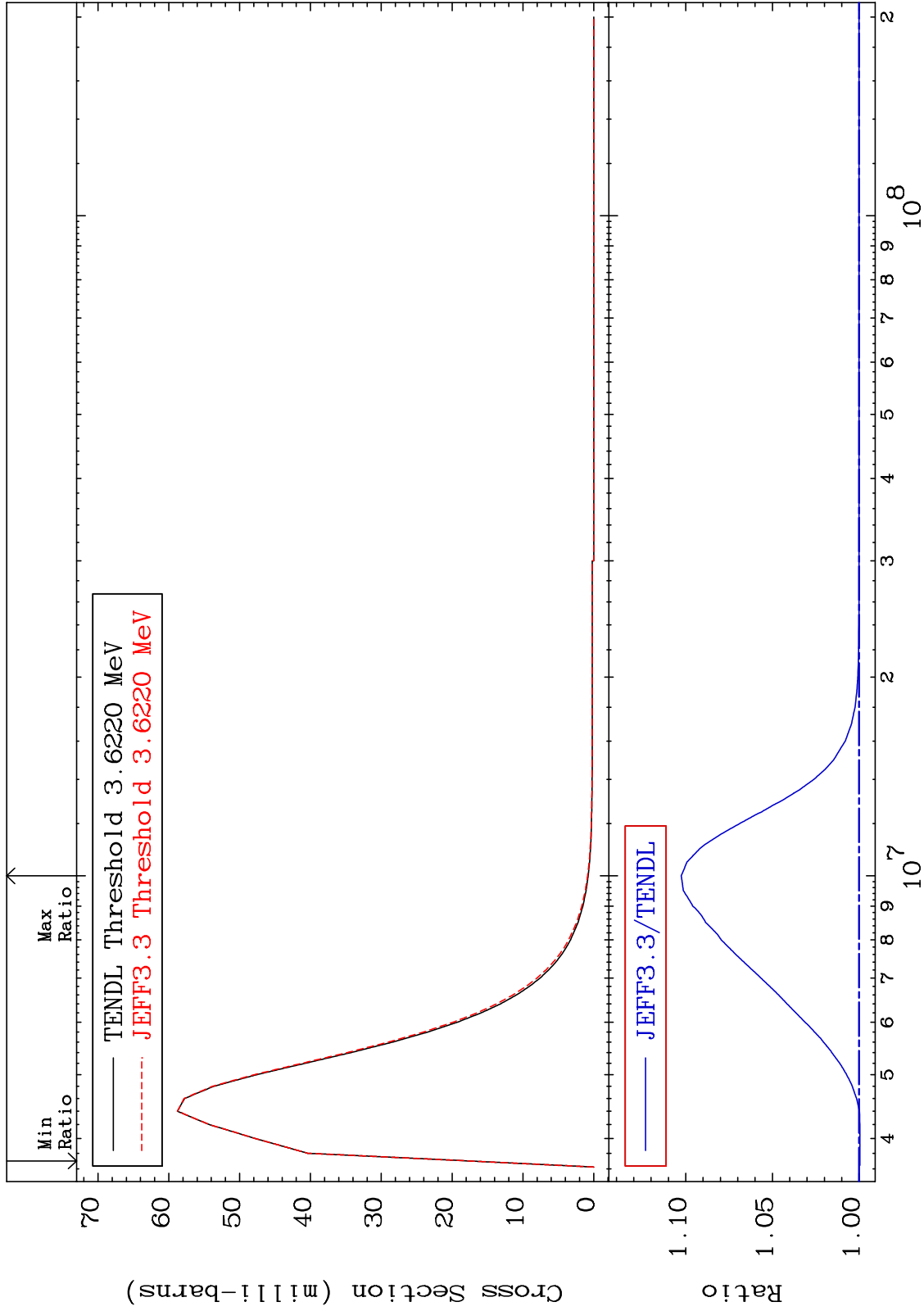
Incident Energy (eV)

26-Fe-60

MAT 2643

MT= 67 (n,n') Level
Cross Section

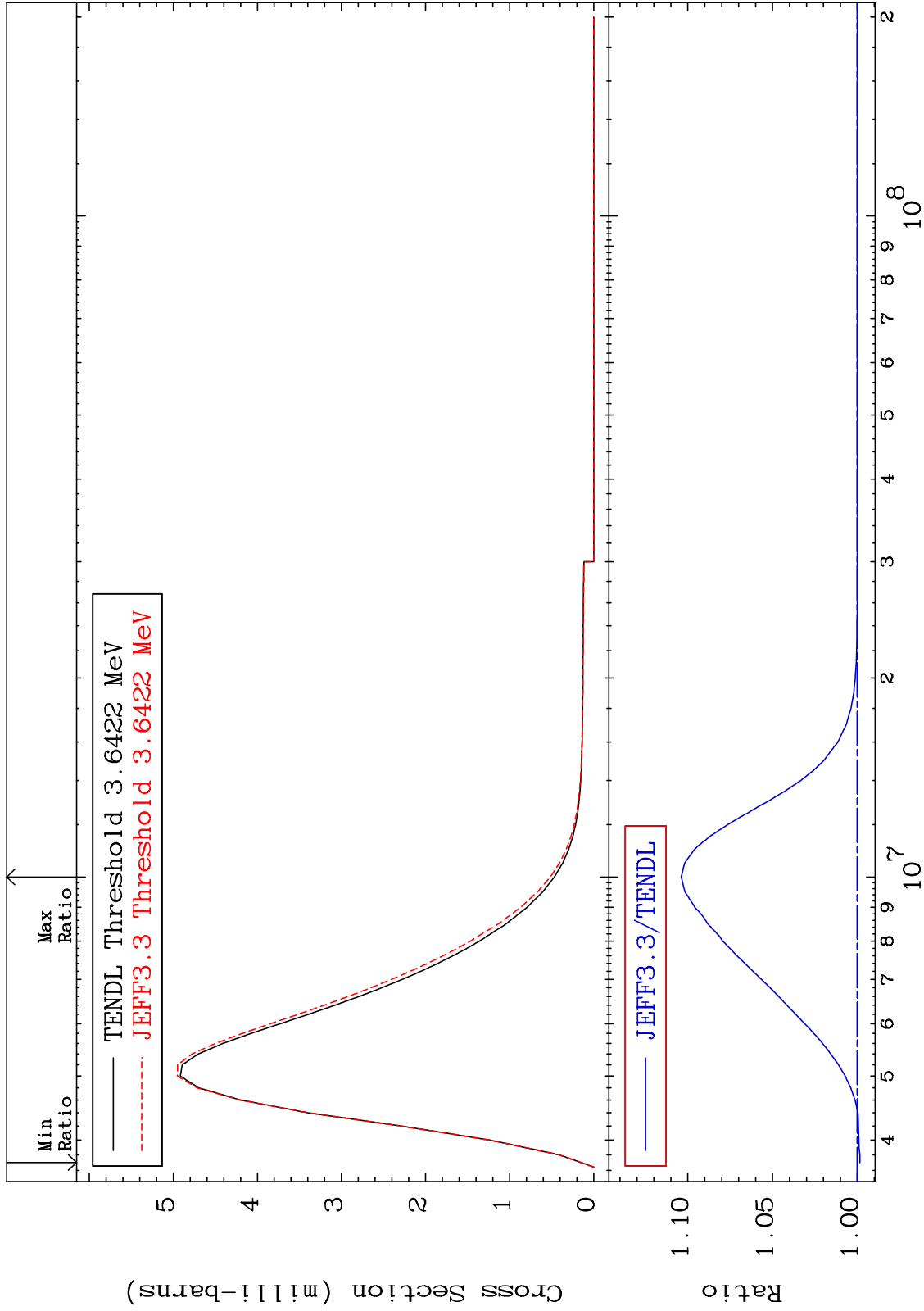
26-Fe-60
-0.051 To 10.26 %



MAT 2643

MT= 68 (n,n') Level
Cross Section

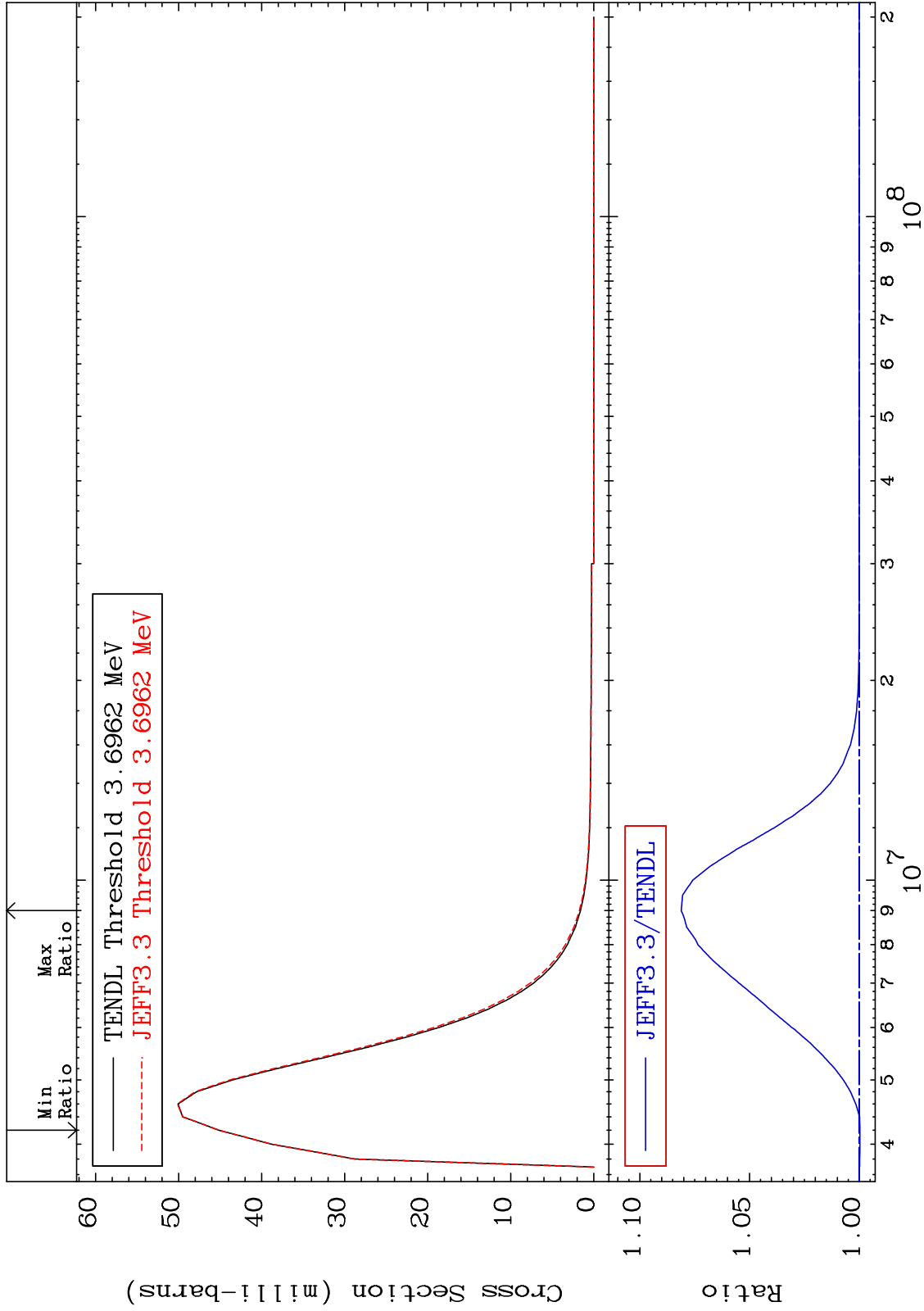
26-Fe-60
-0.153 To 10.39 %



MAT 2643

MT= 69 (n,n') Level
Cross Section

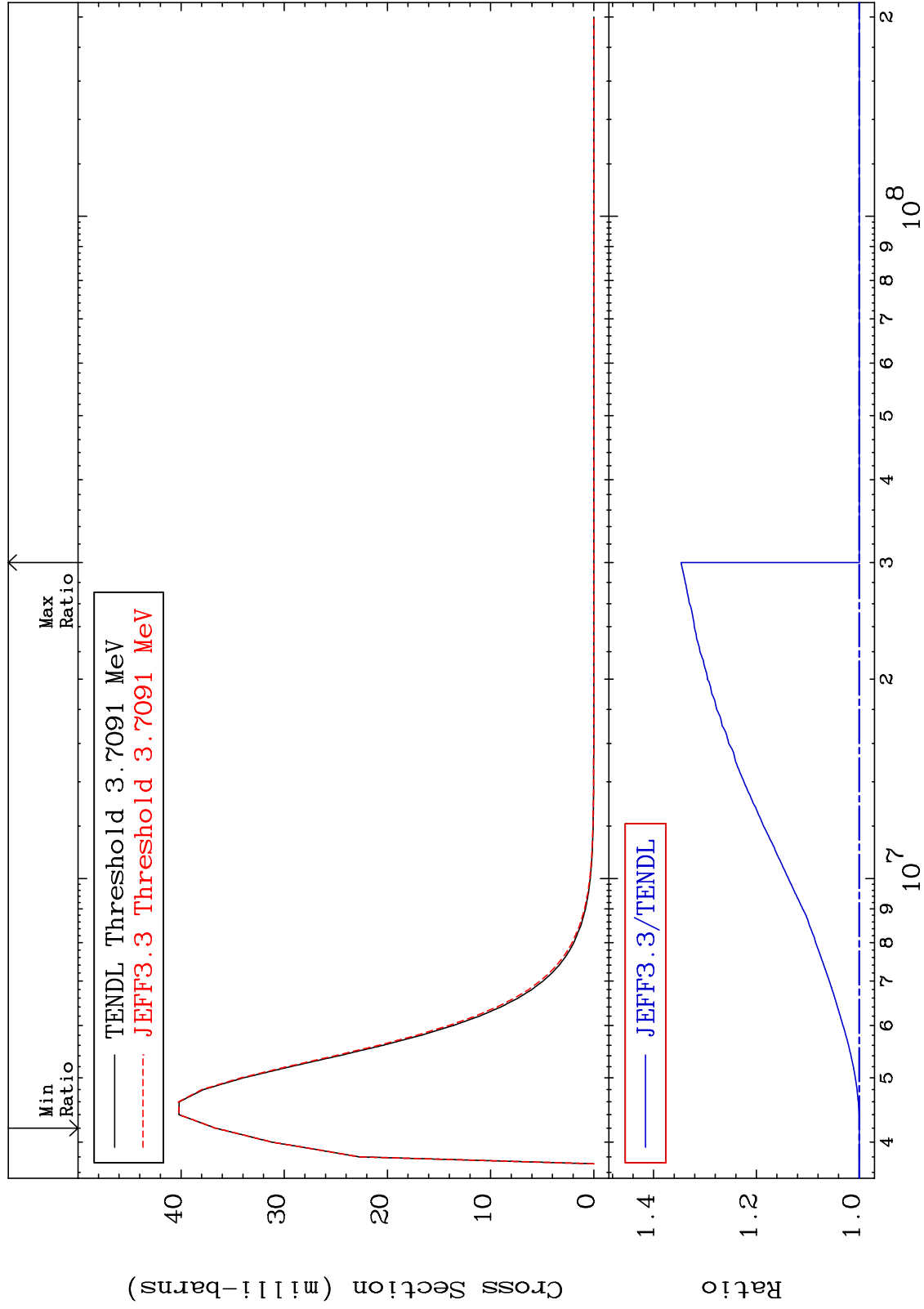
26-Fe-60
-0.031 To 8.112 %



MAT 2643

MT= 70 (n,n') Level
Cross Section

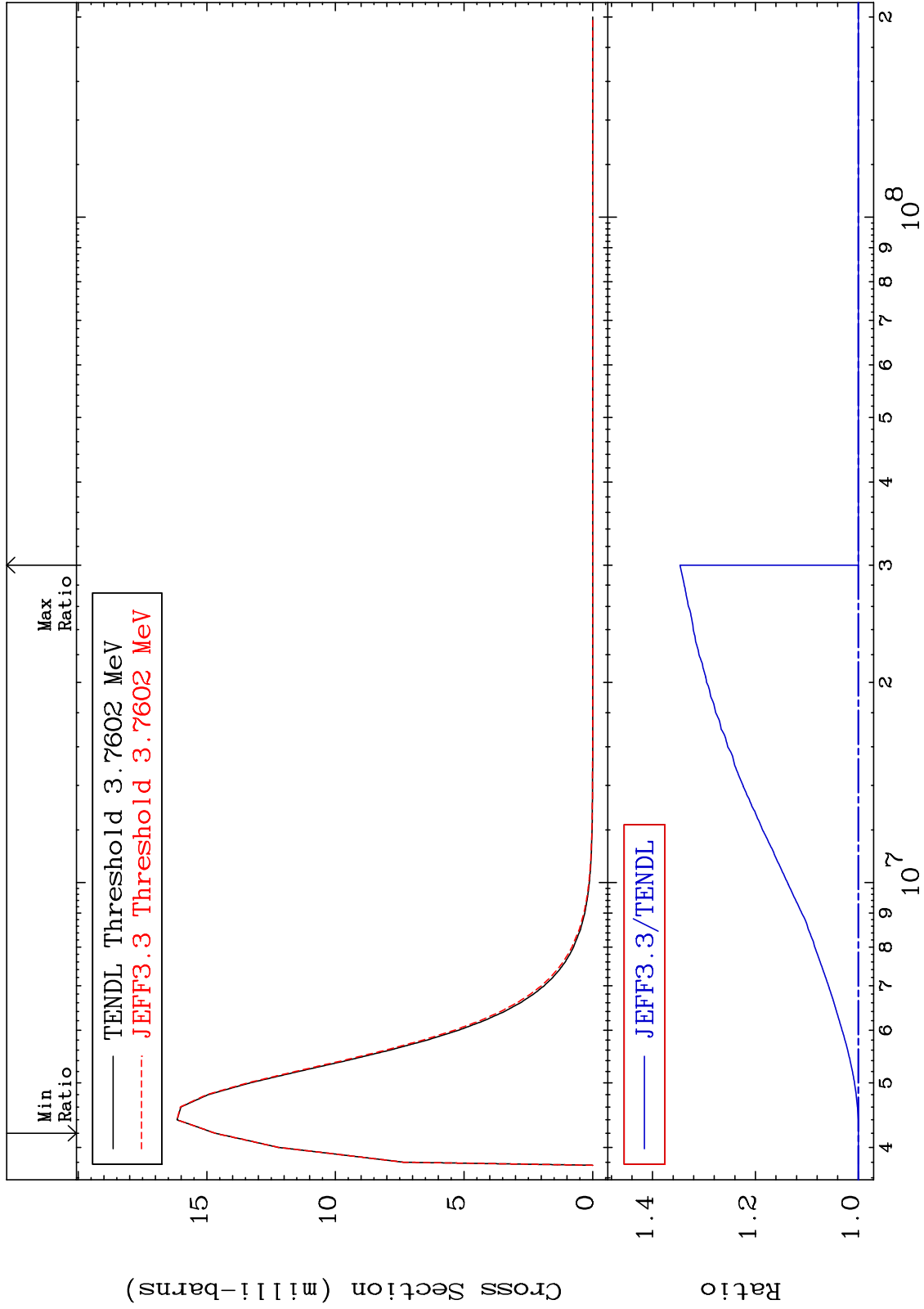
26-Fe-60
-0.030 To 34.66 %



MAT 2643

MT= 71 (n,n') Level
Cross Section

26-Fe-60
-0.029 To 34.66 %



40

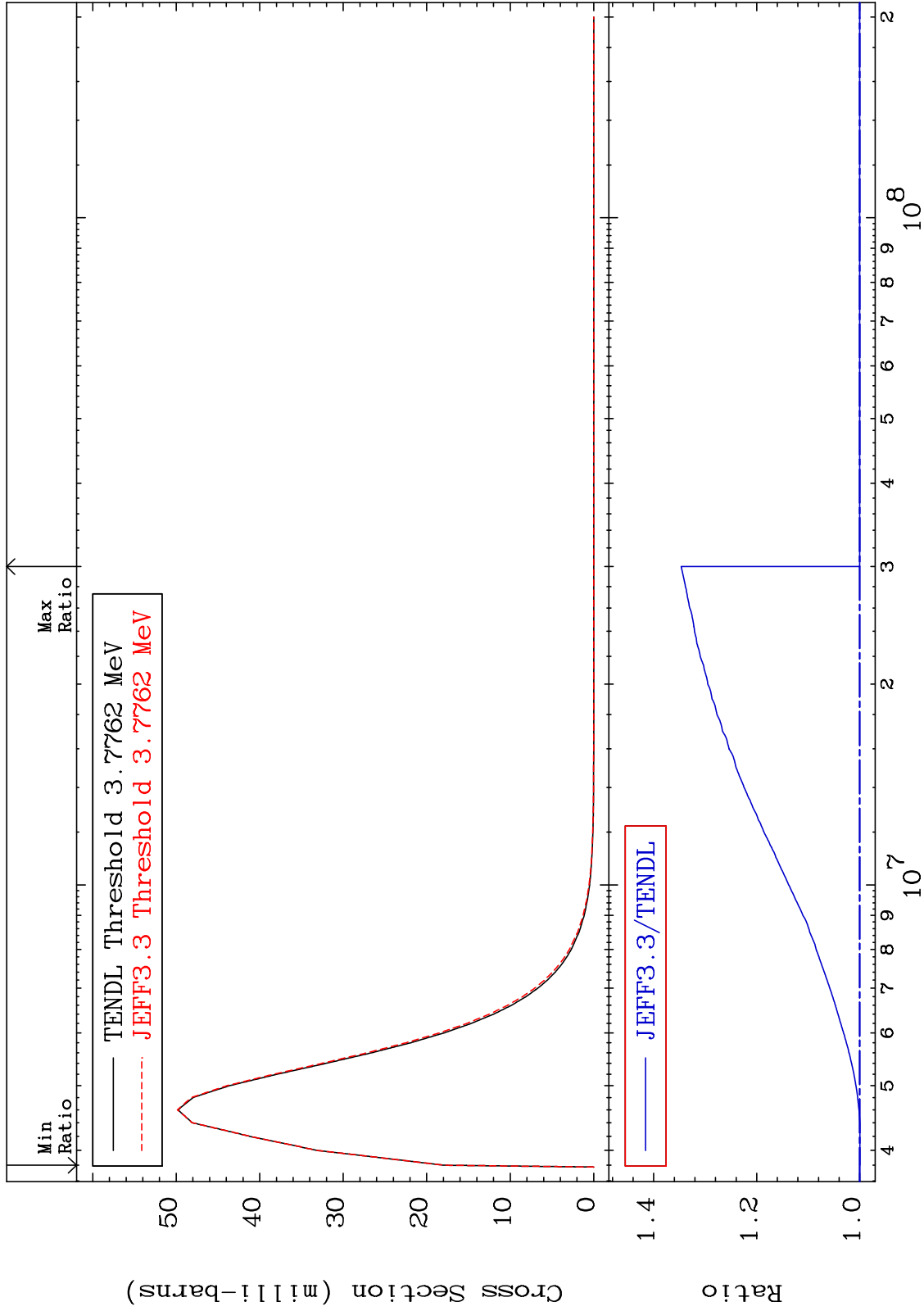
Incident Energy (eV)

26-Fe-60

MAT 2643

MT= 72 (n,n') Level
Cross Section

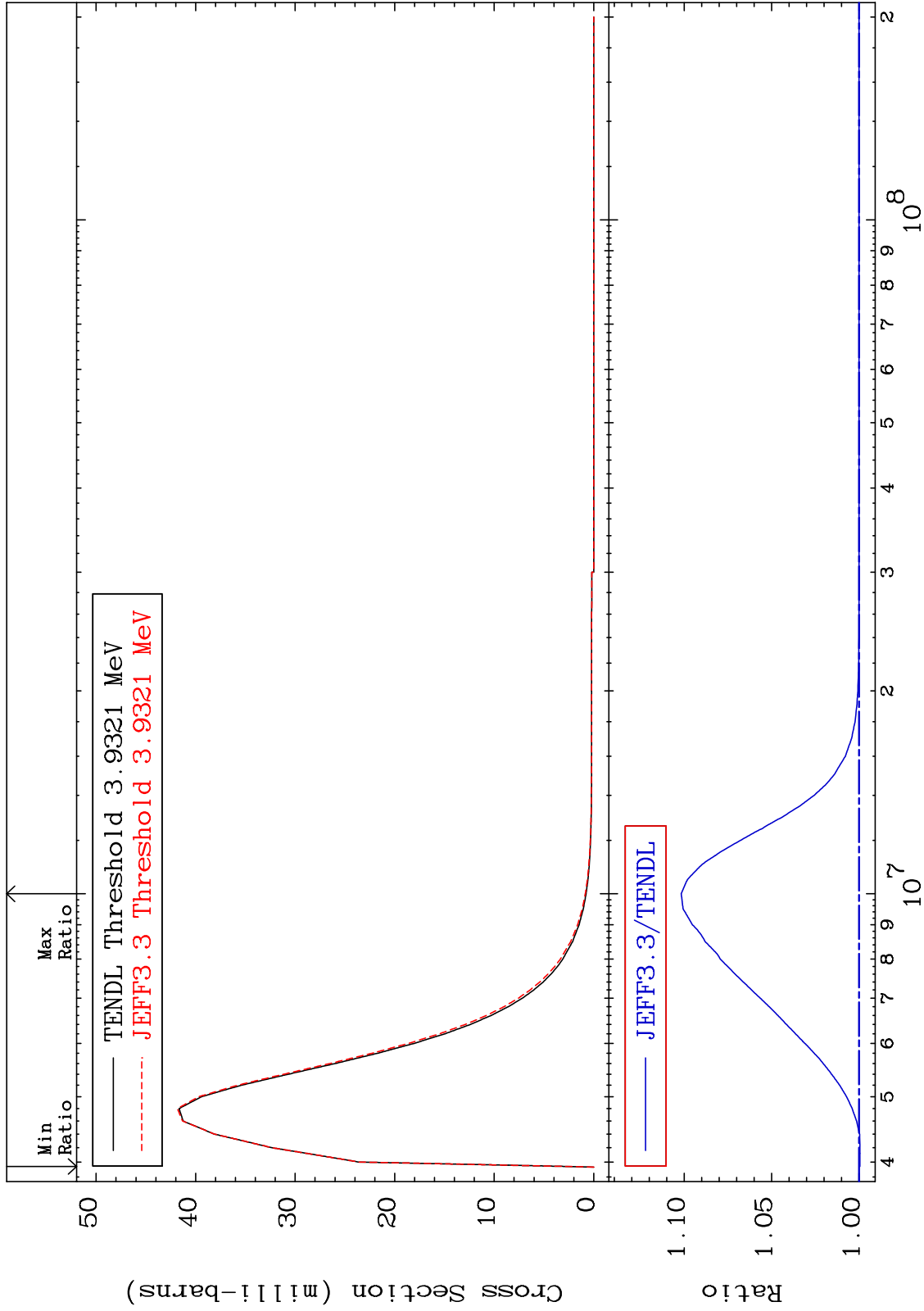
26-Fe-60
-0.049 To 34.66 %



MAT 2643

MT= 73 (n,n') Level
Cross Section

26-Fe-60
-0.050 To 10.17 %



42

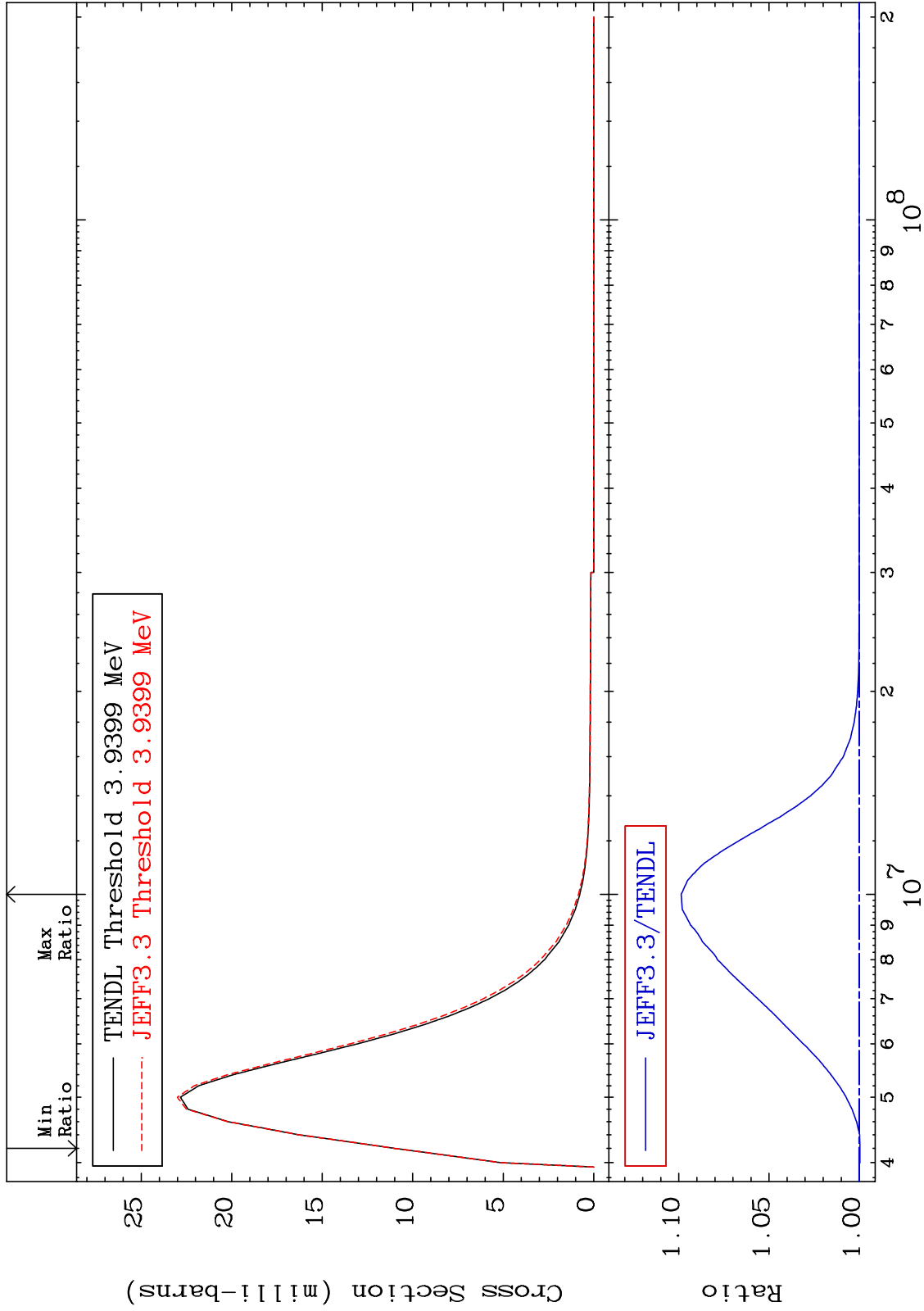
Incident Energy (eV)

26-Fe-60

MAT 2643

MT= 74 (n,n') Level
Cross Section

26-Fe-60
-0.037 To 9.868 %



43

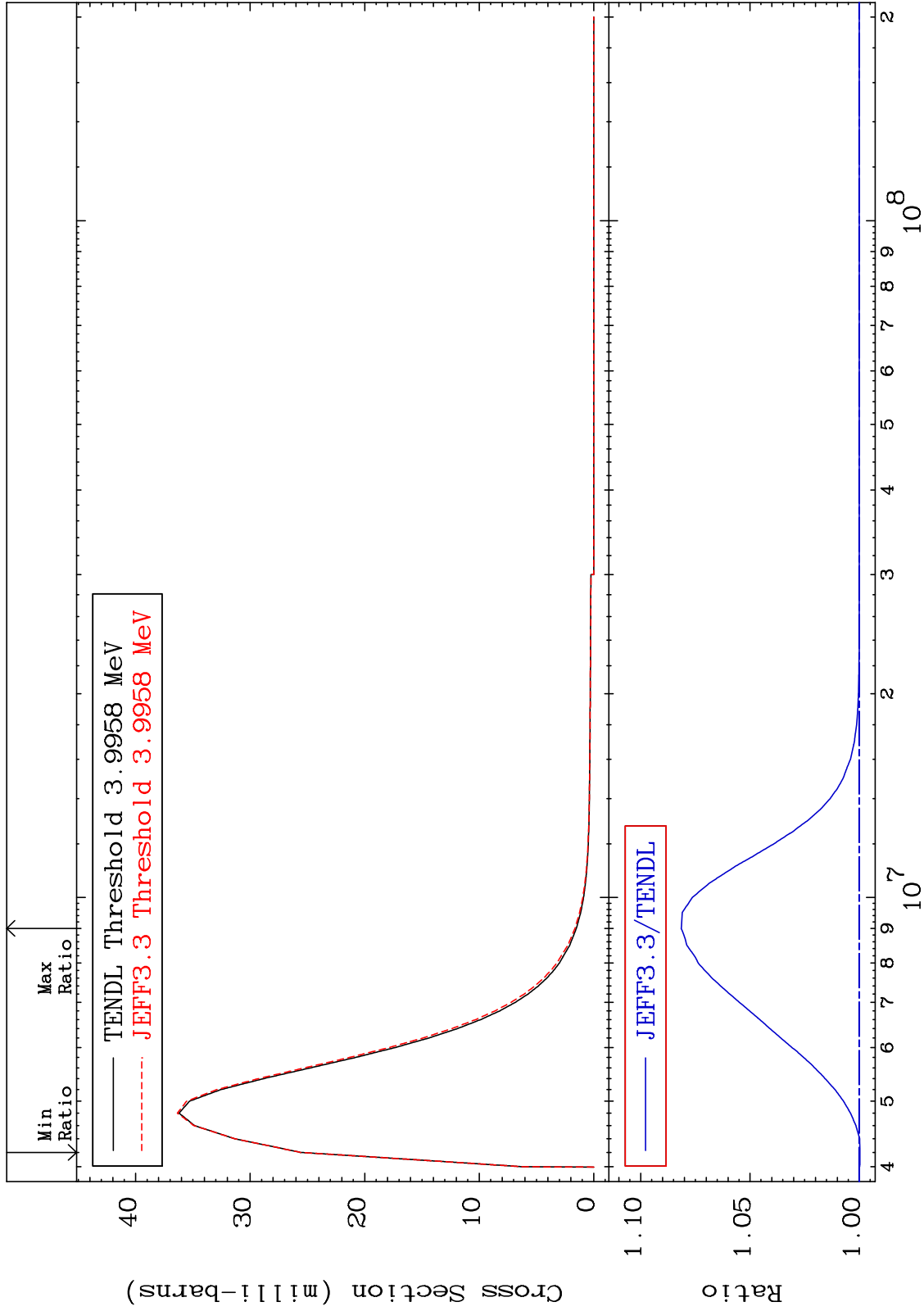
Incident Energy (eV)

26-Fe-60

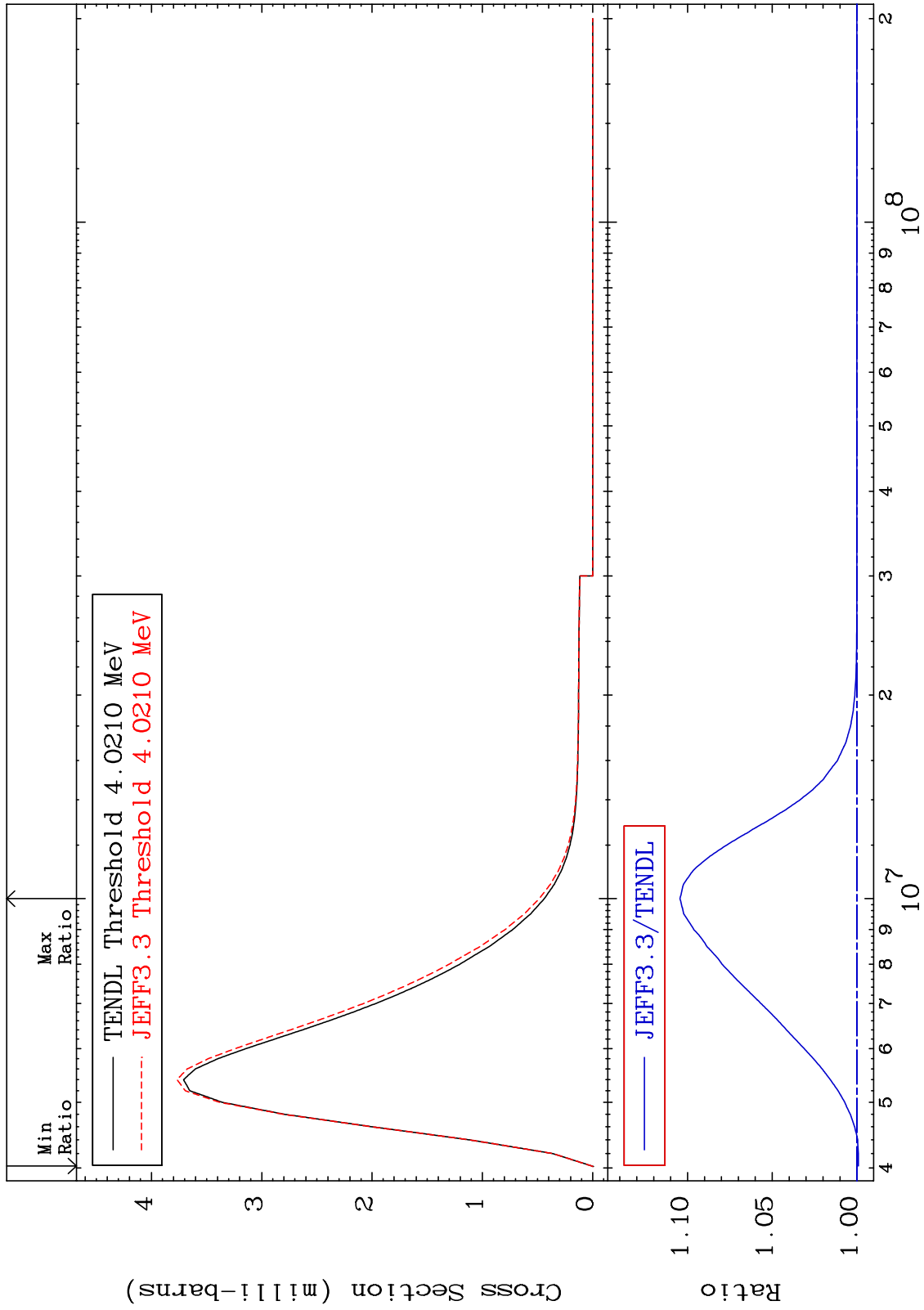
MAT 2643

MT= 75 (n,n') Level
Cross Section

26-Fe-60
-0.028 To 8.151 %



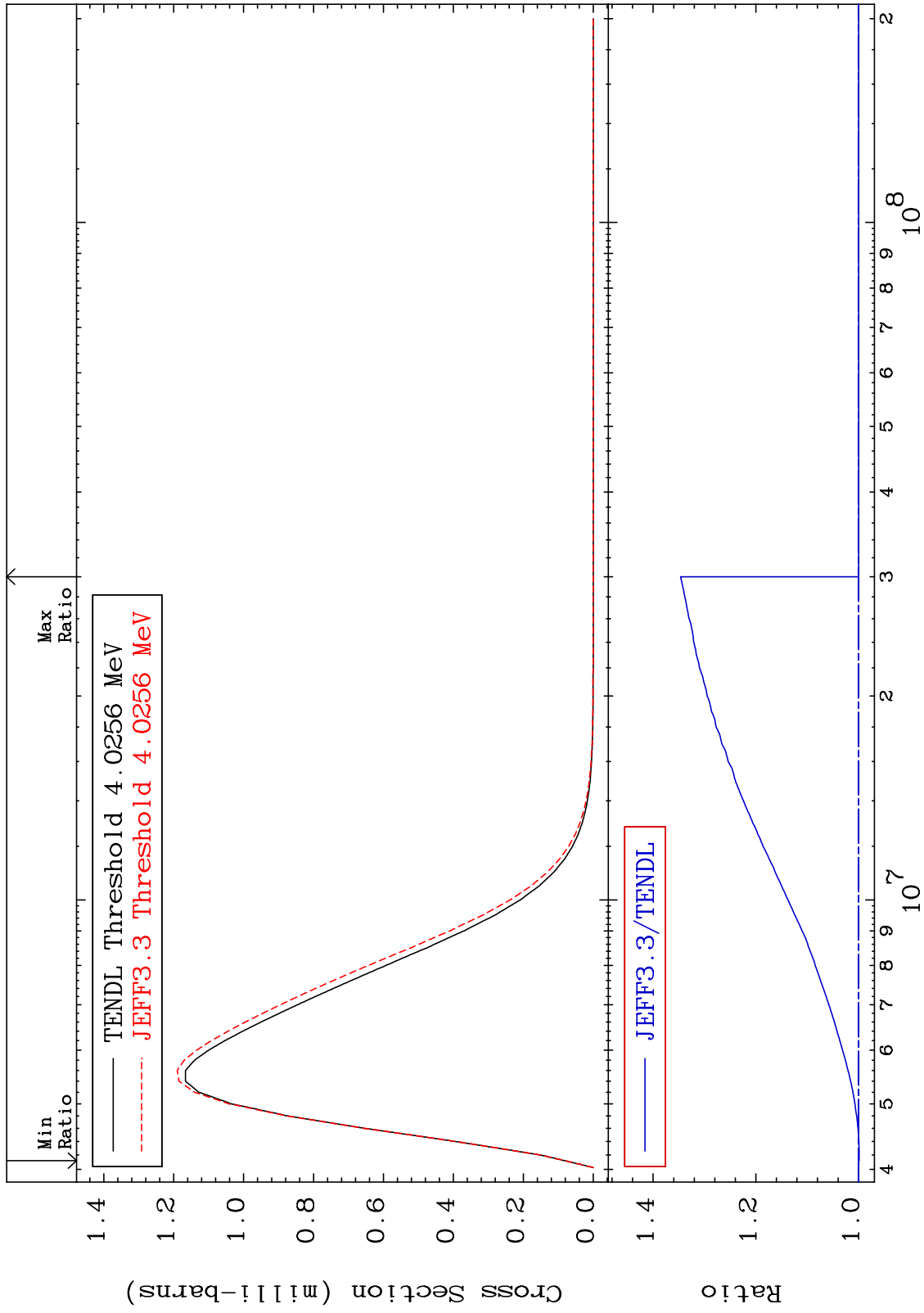
MAT 2643 MT= 76 (n,n') Level
Cross Section 26-Fe-60
-0.089 To 10.45 %



MAT 2643

MT= 77 (n,n') Level
Cross Section

26-Fe-60
-0.108 To 34.65 %



46

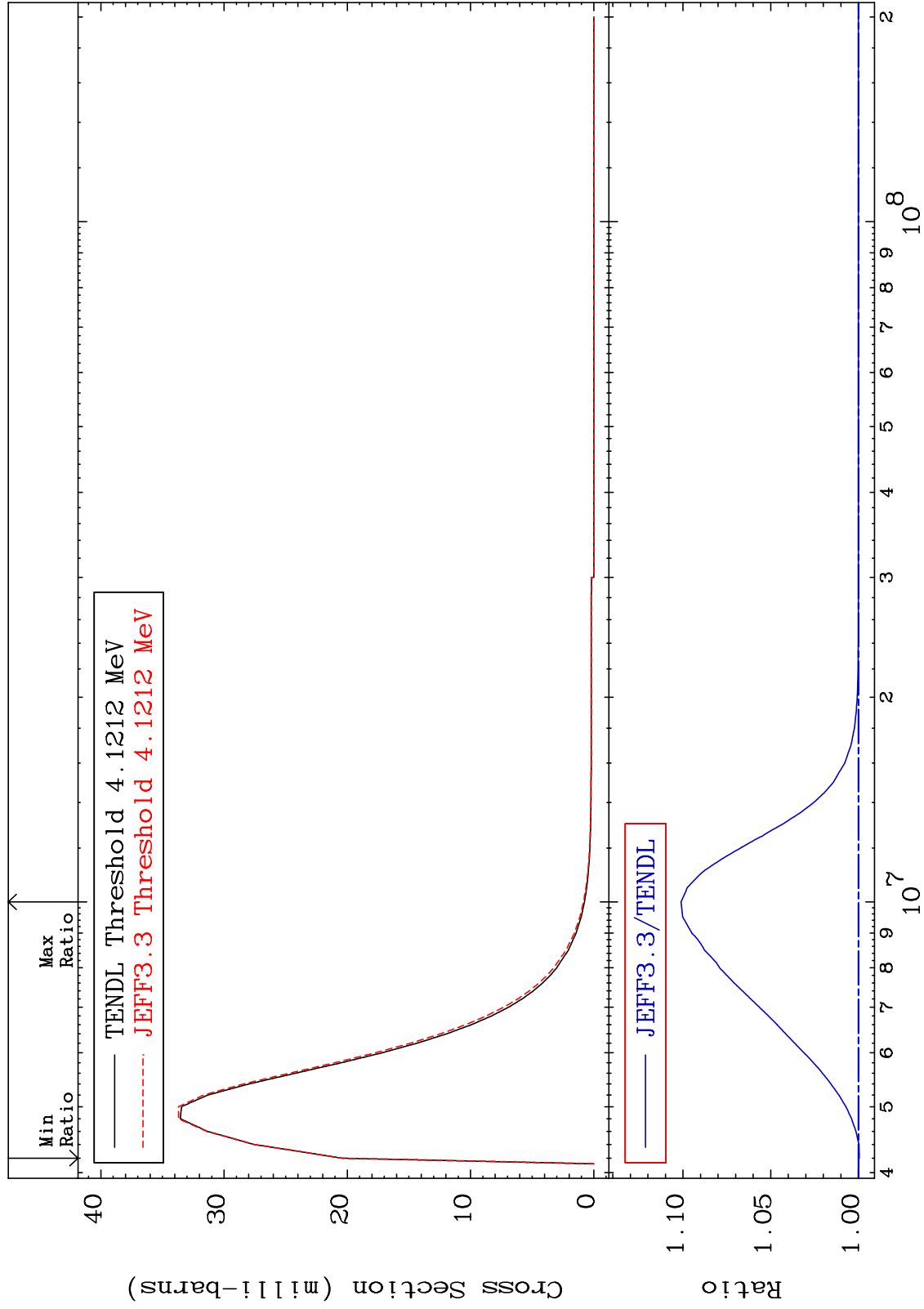
Incident Energy (eV)

26-Fe-60

MAT 2643

MT= 78 (n,n') Level
Cross Section

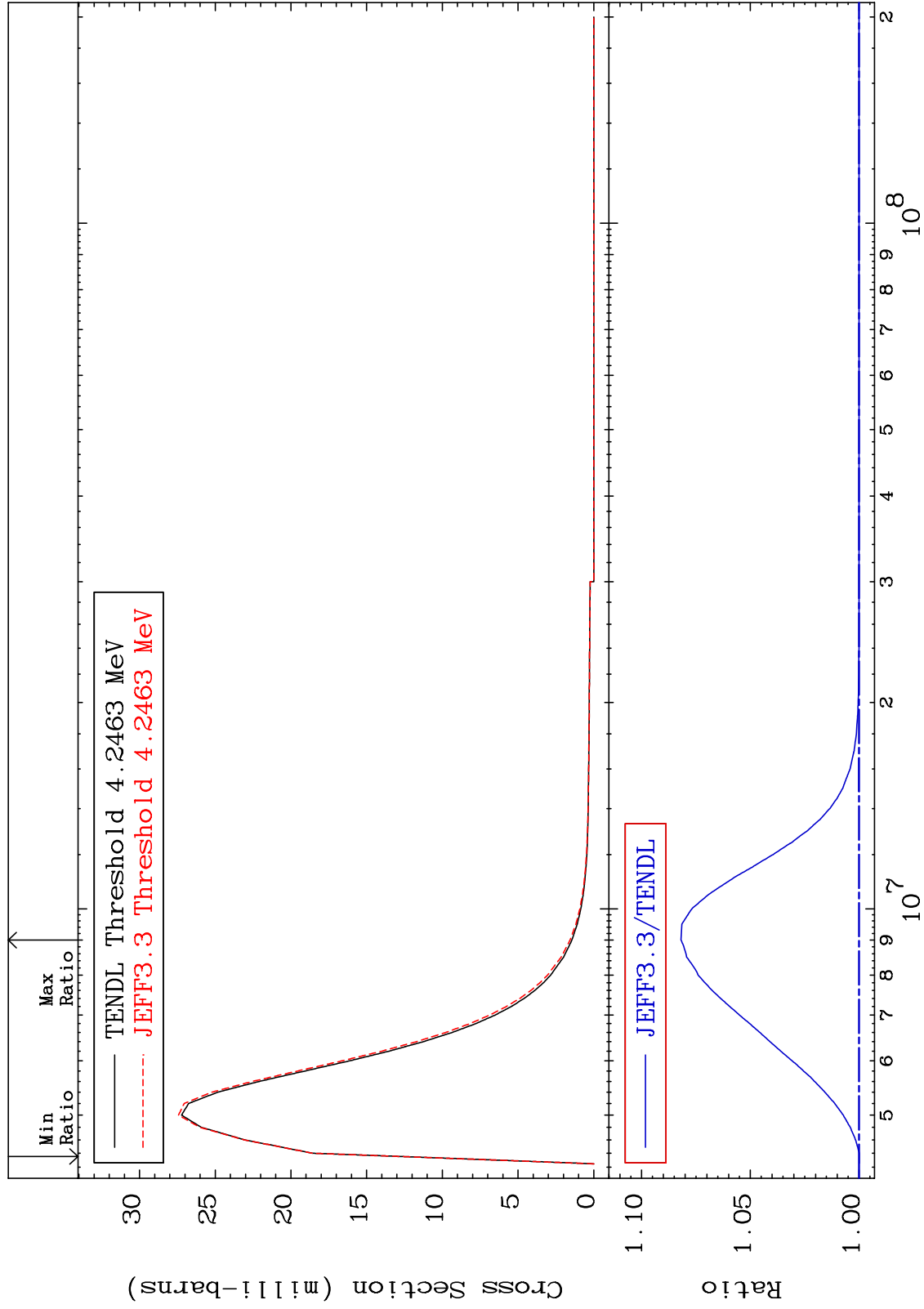
26-Fe-60
-0.048 To 10.11 %



MAT 2643

MT= 79 (n,n') Level
Cross Section

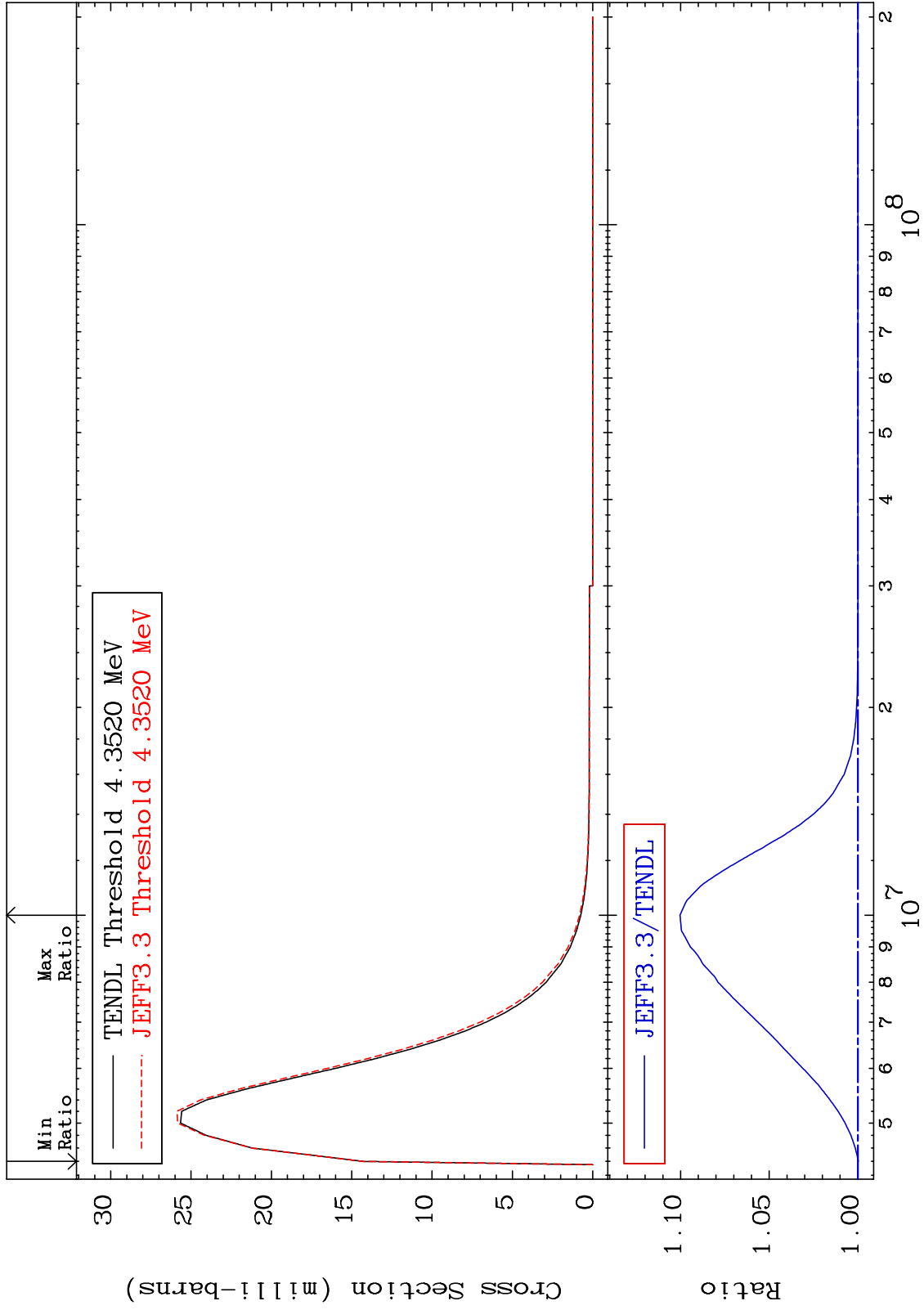
26-Fe-60
-0.011 To 8.187 %



MAT 2643

MT= 80 (n,n') Level
Cross Section

26-Fe-60
-0.031 To 10.03 %



49

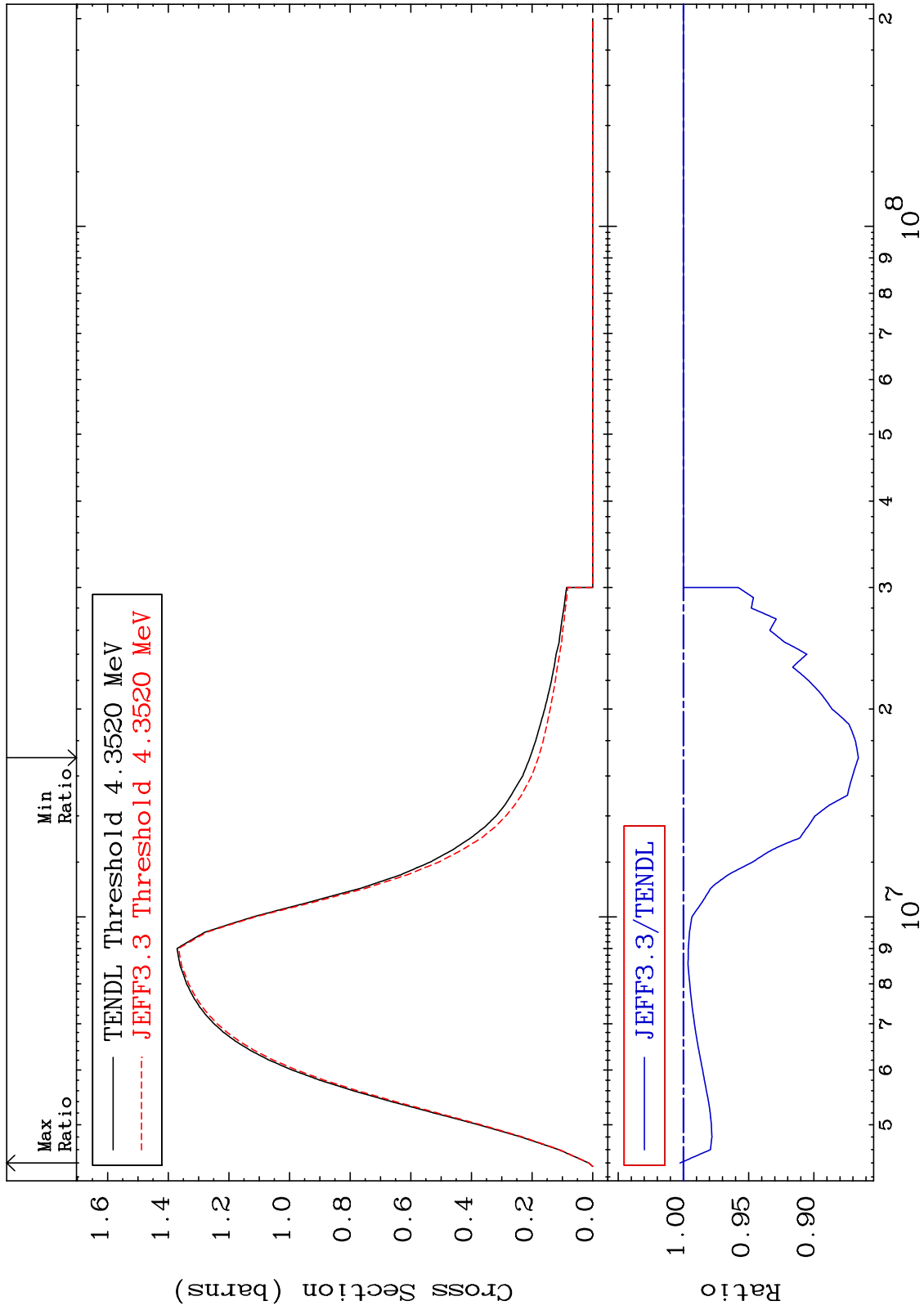
Incident Energy (eV)

26-Fe-60

MAT 2643

(n,n') Continuum
Cross Section

26-Fe-60
-13.41 To 0.260 %



50

Incident Energy (eV)

26-Fe-60

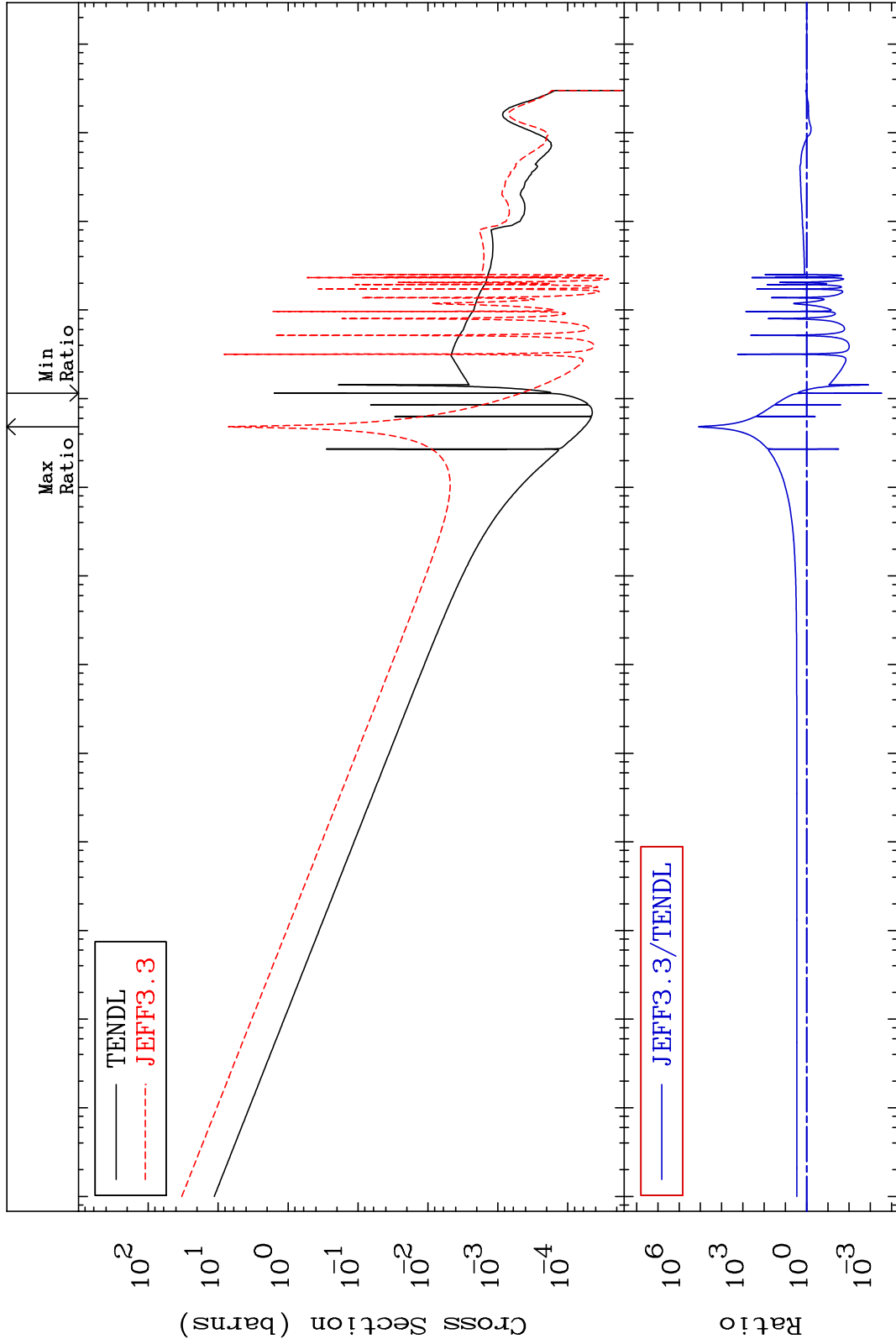
MAT 2643

(n, γ)

26-Fe-60

Cross Section

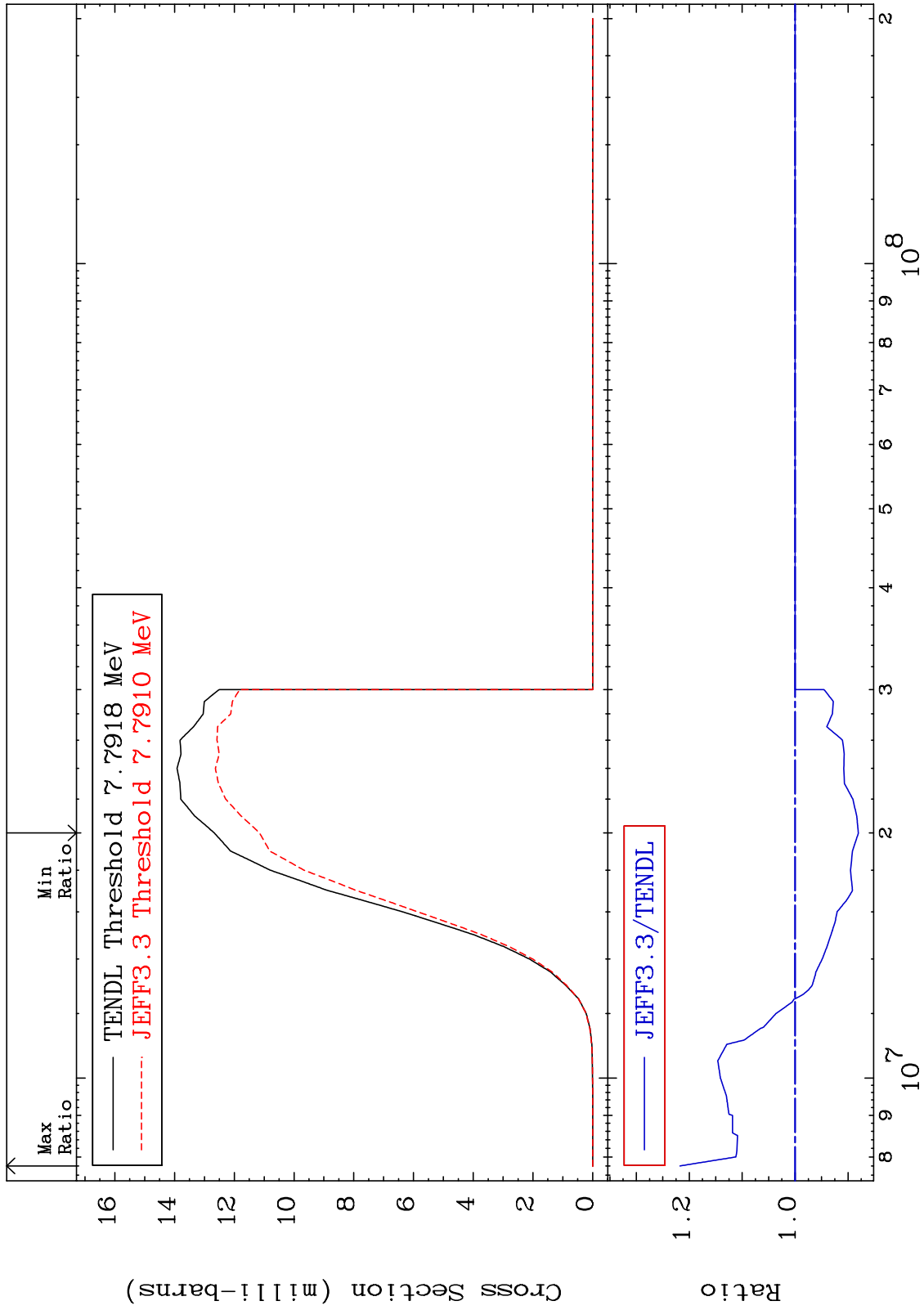
-99.97 To 9999. %



Incident Energy (eV)

26-Fe-60

MAT 2643 (n,p) Cross Section 26-Fe-60 -11.97 To 21.74 %



52 Incident Energy (eV) 26-Fe-60

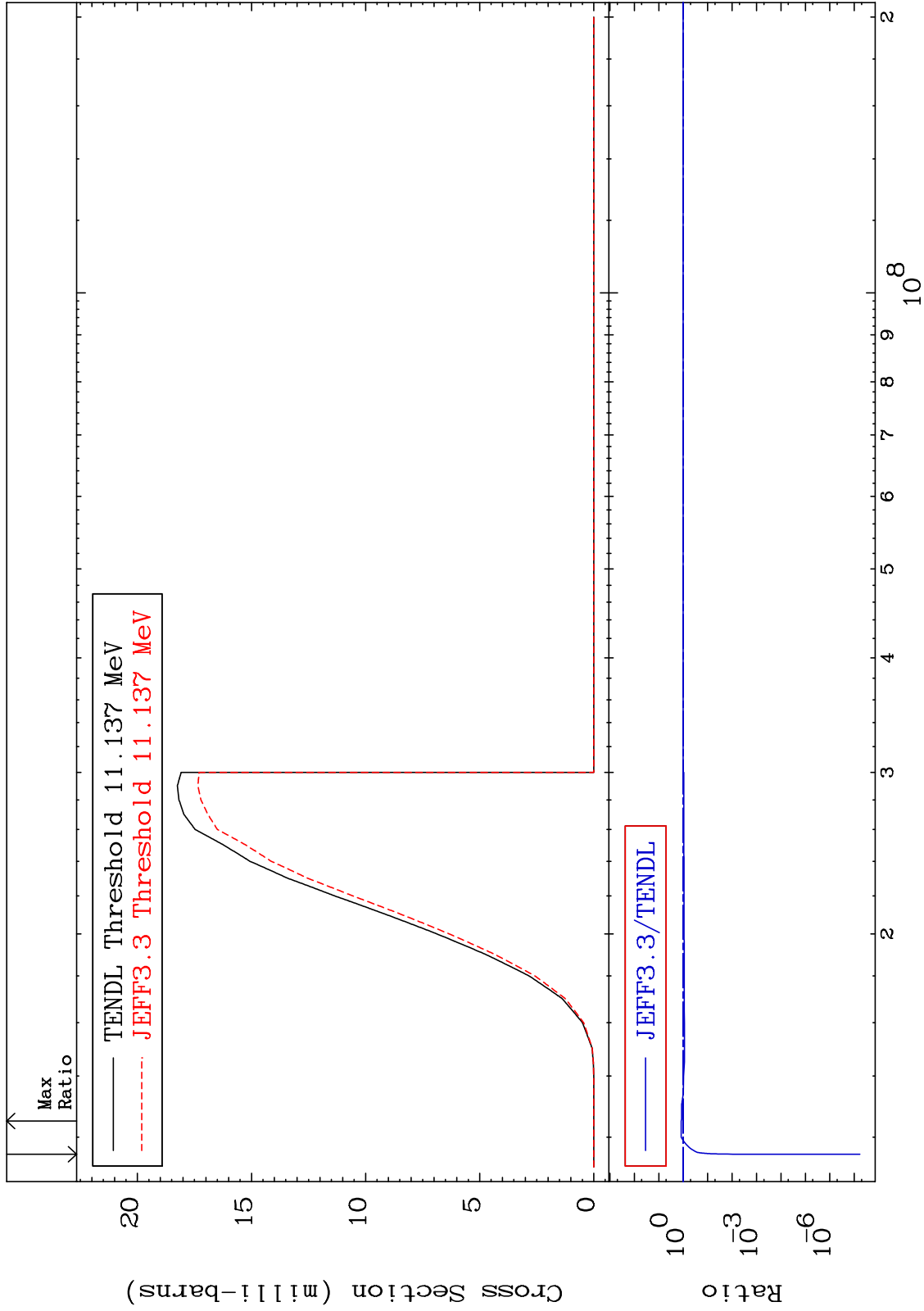
MAT 2643

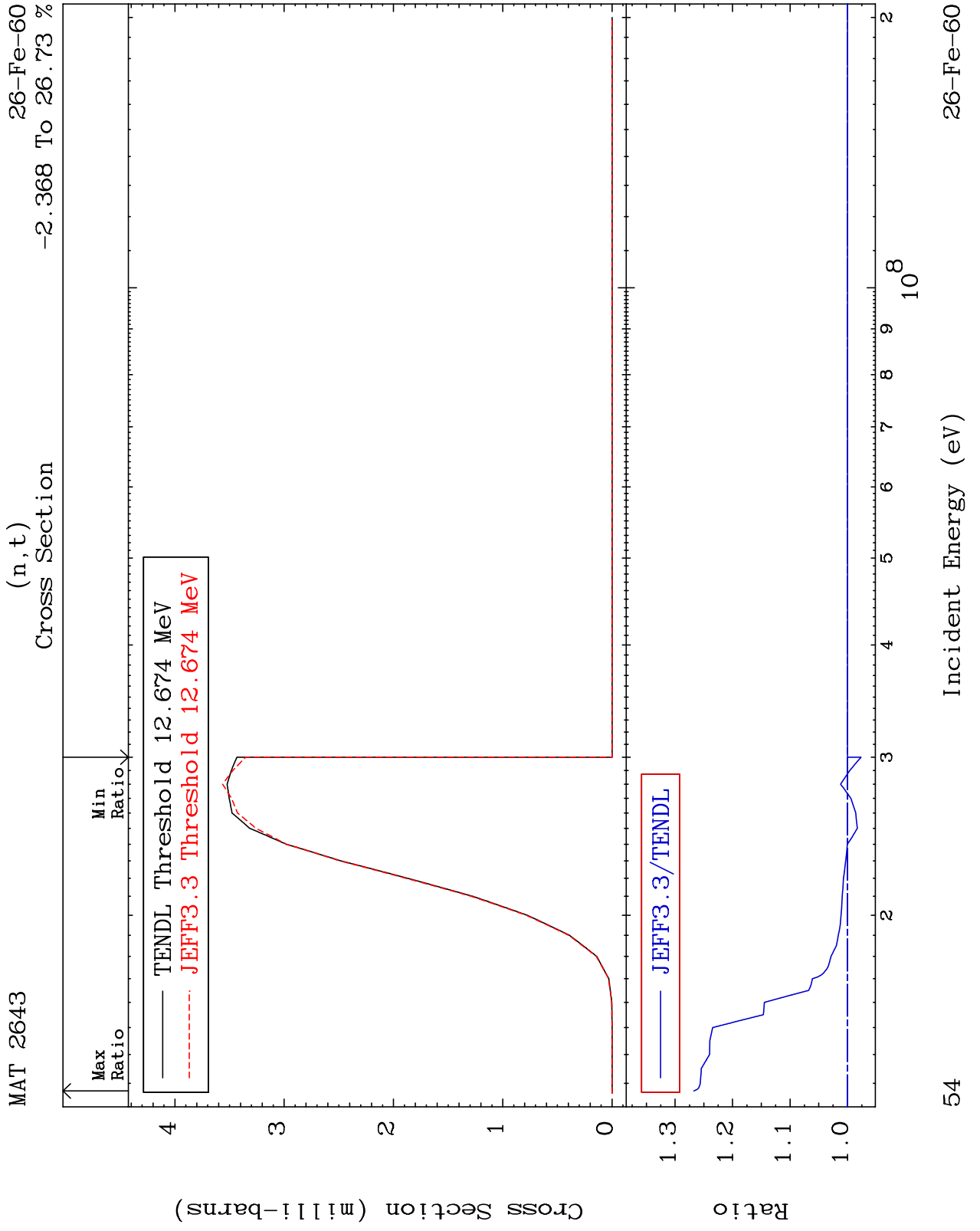
(n, d)

²⁶Fe-60

Cross Section

-100.0 To 20.66 %





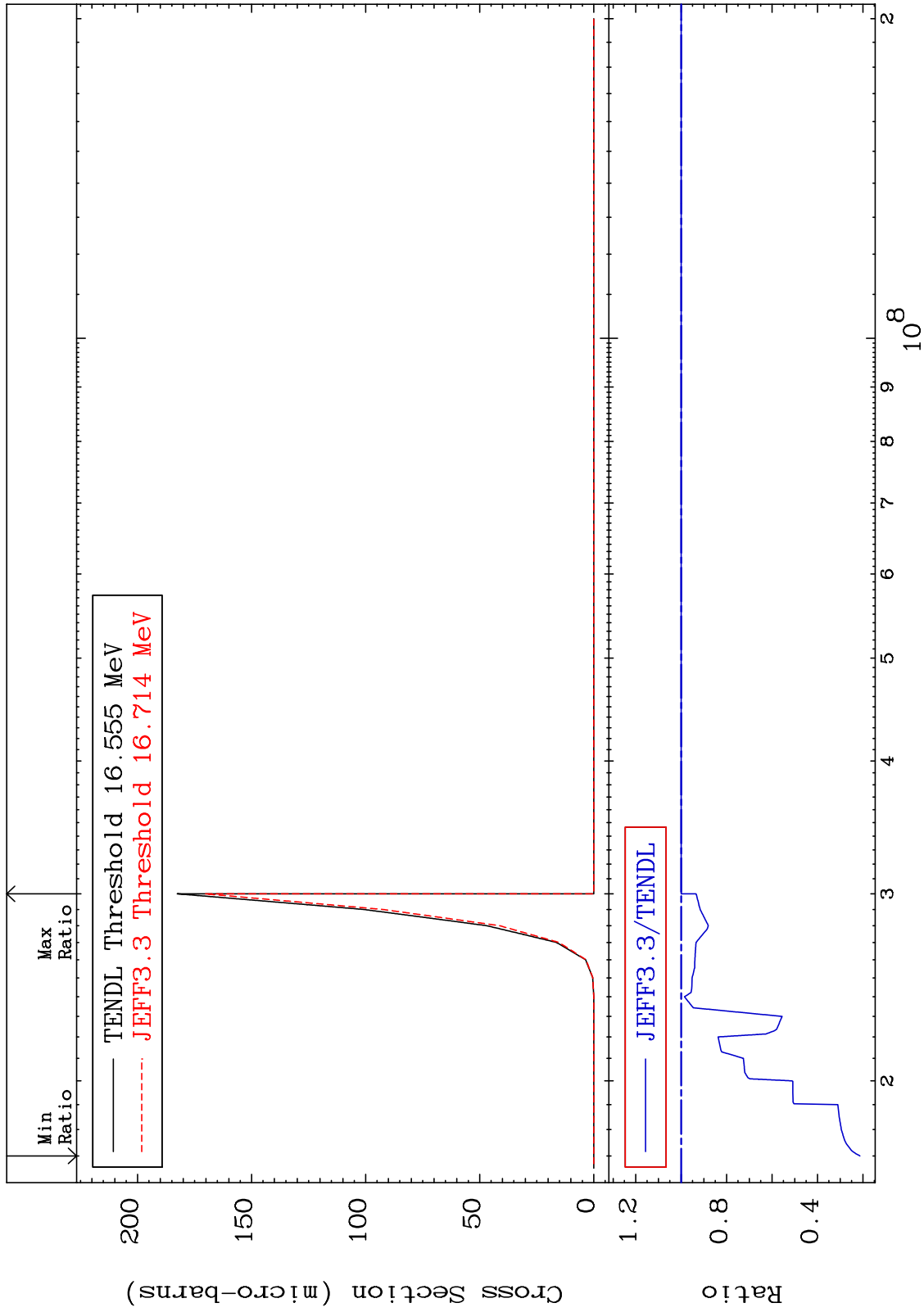
MAT 2643

(n, He-3)

²⁶Fe-60

Cross Section

-78.70 To 0.000 %



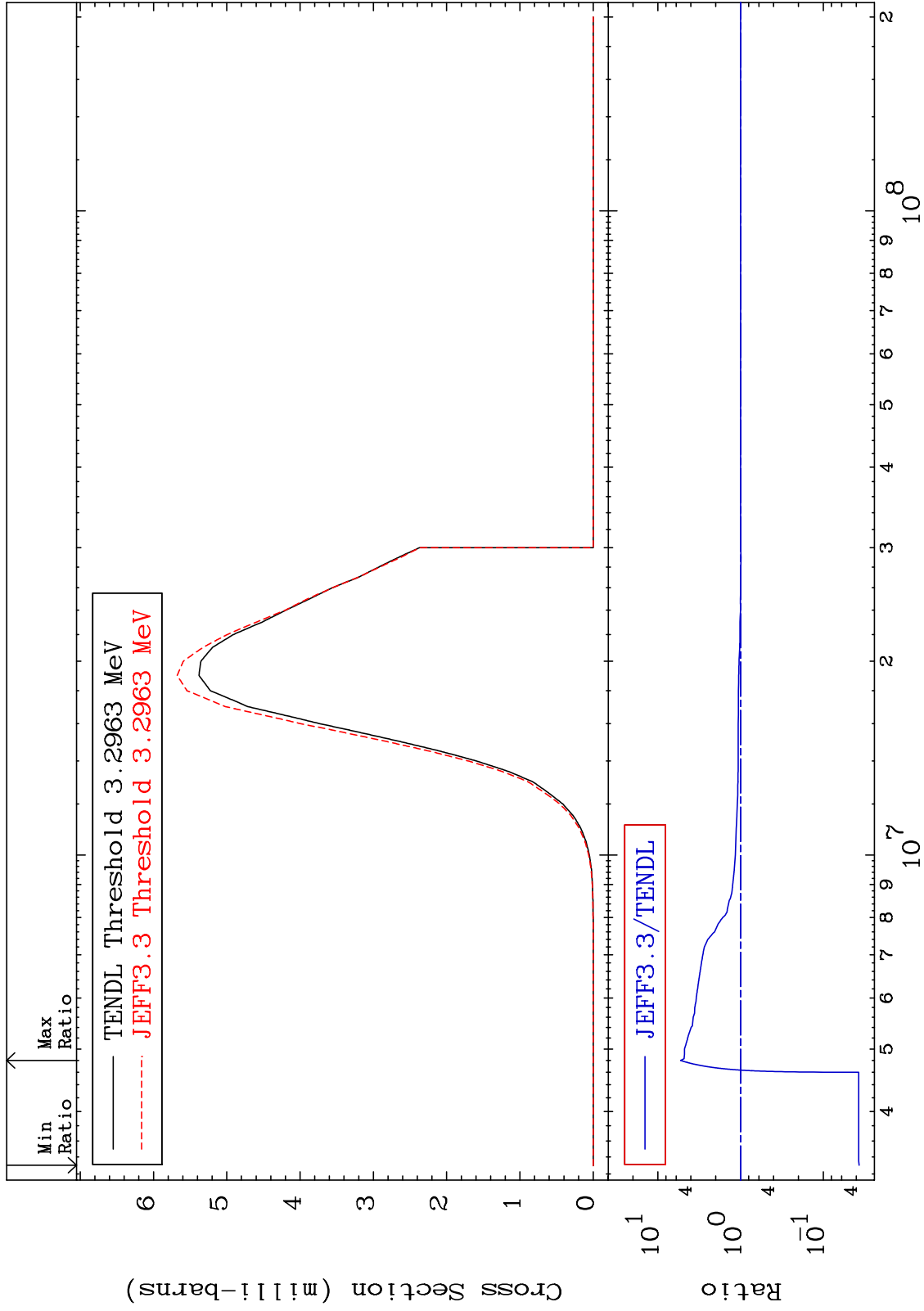
MAT 2643

(n, α)

²⁶Fe-60

Cross Section

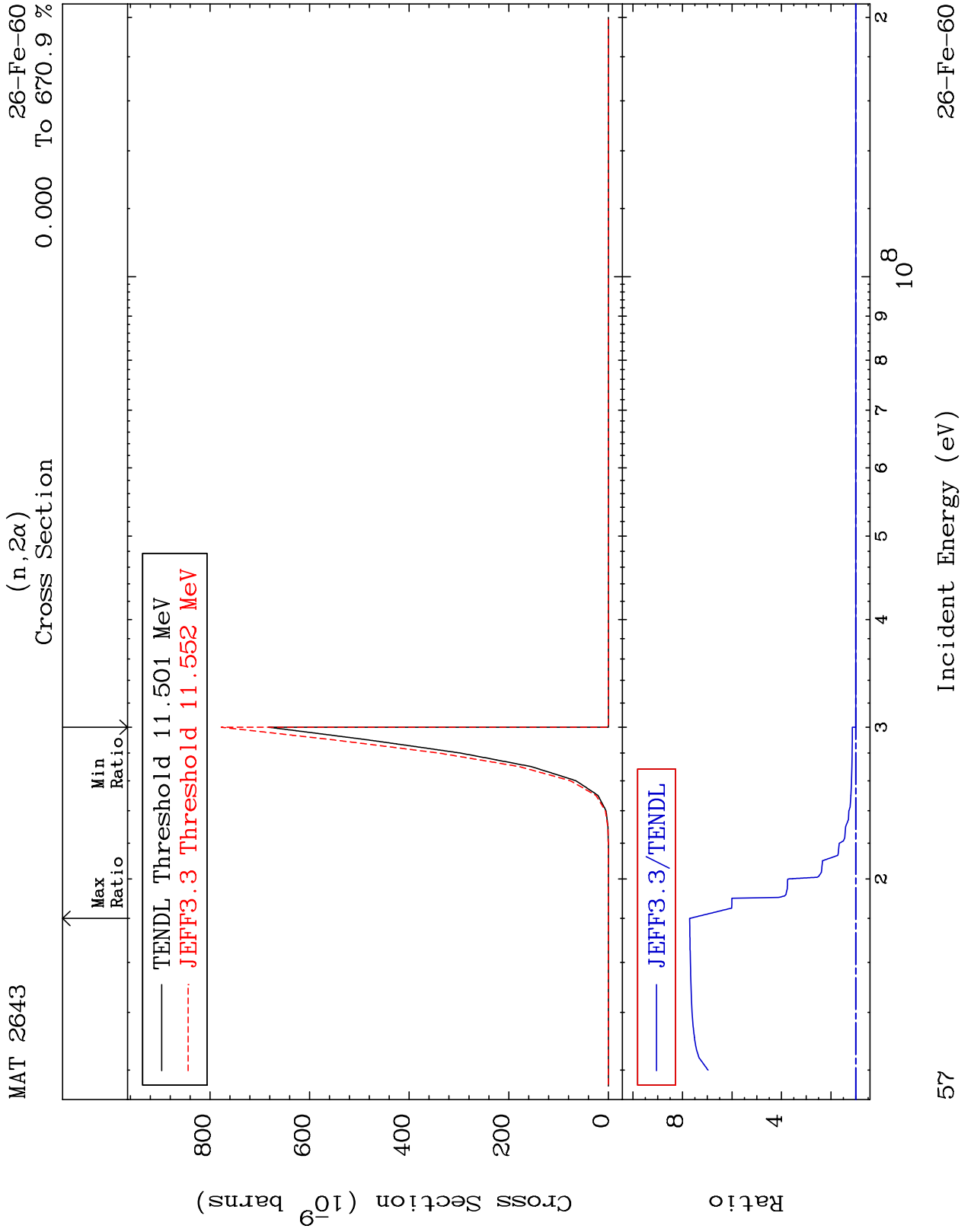
-96.31 To 431.2 %



56

Incident Energy (eV)

²⁶Fe-60



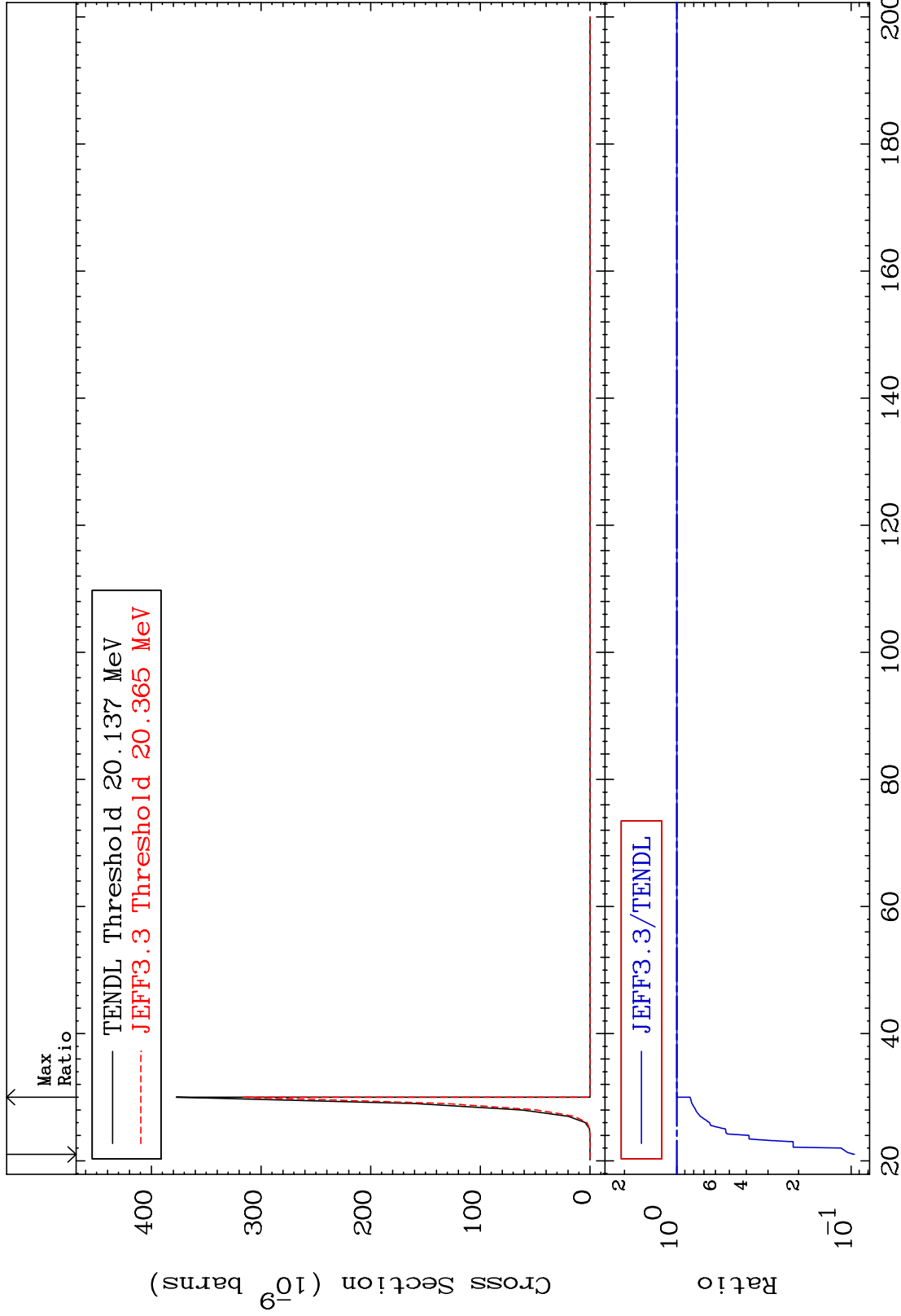
MAT 2643

(n,2p)

²⁶Fe-60

Cross Section

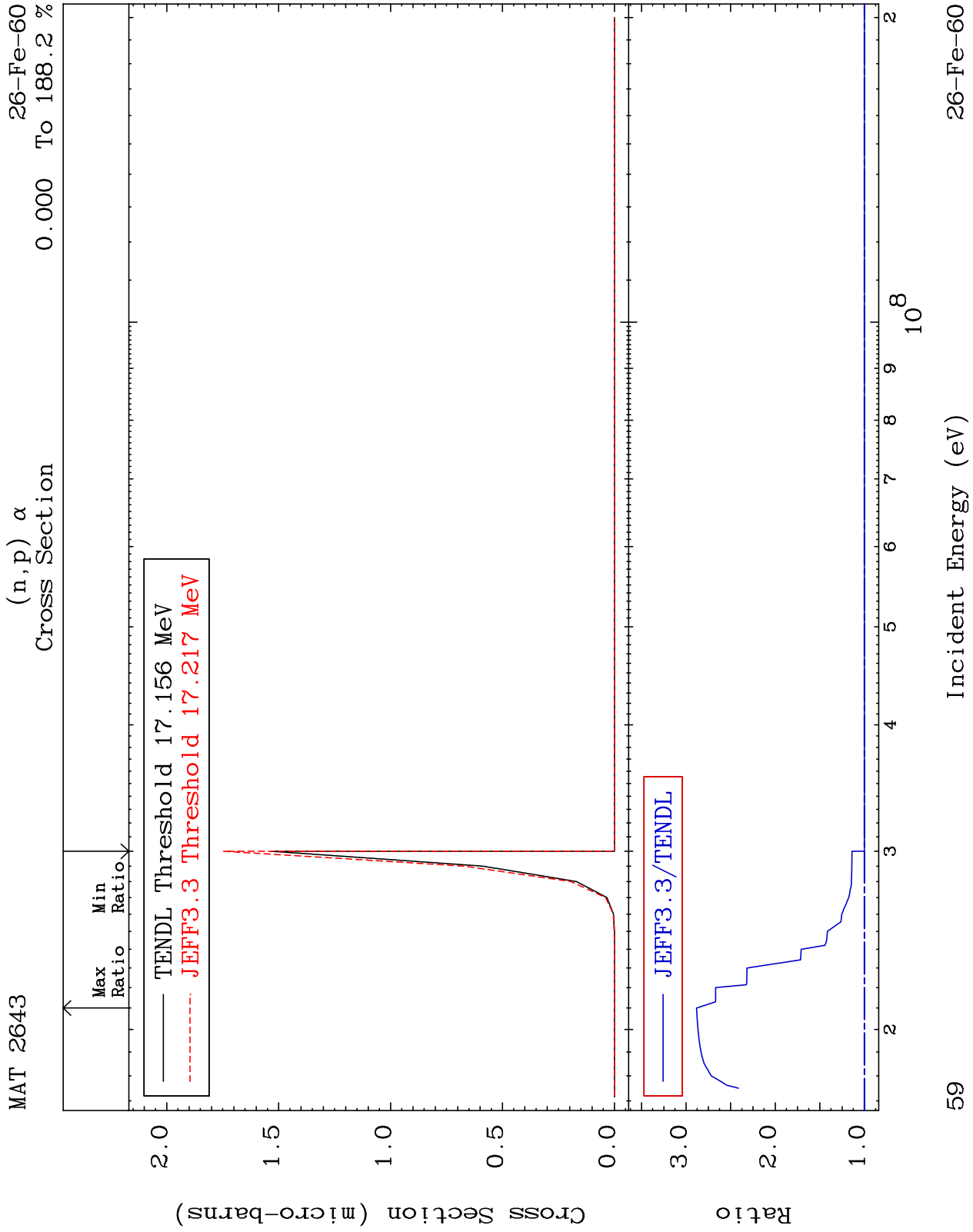
-90.43 To 0.000 %



Incident Energy (MeV)

²⁶Fe-60

58



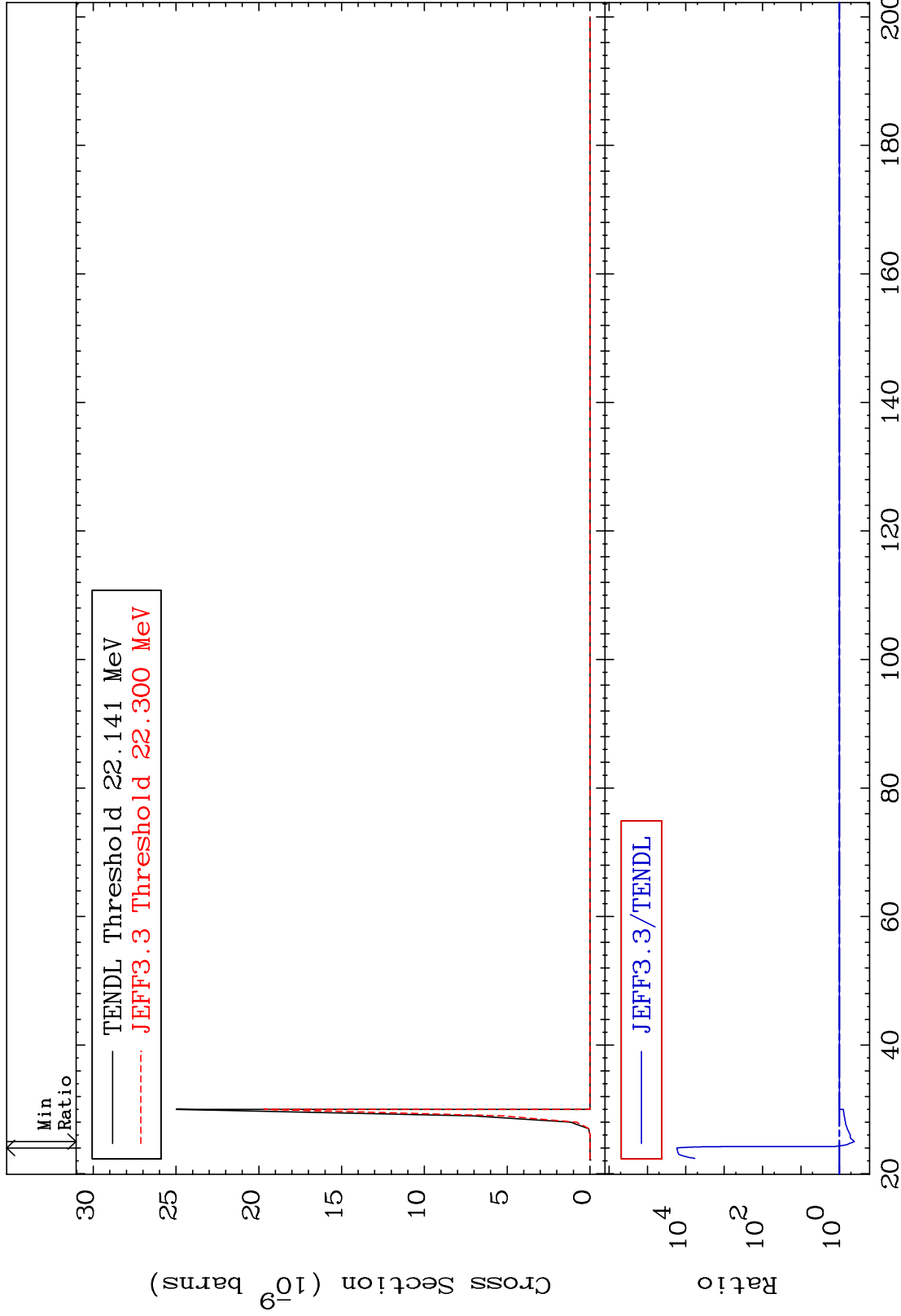
MAT 2643

(n,p) d

²⁶Fe-60

Cross Section

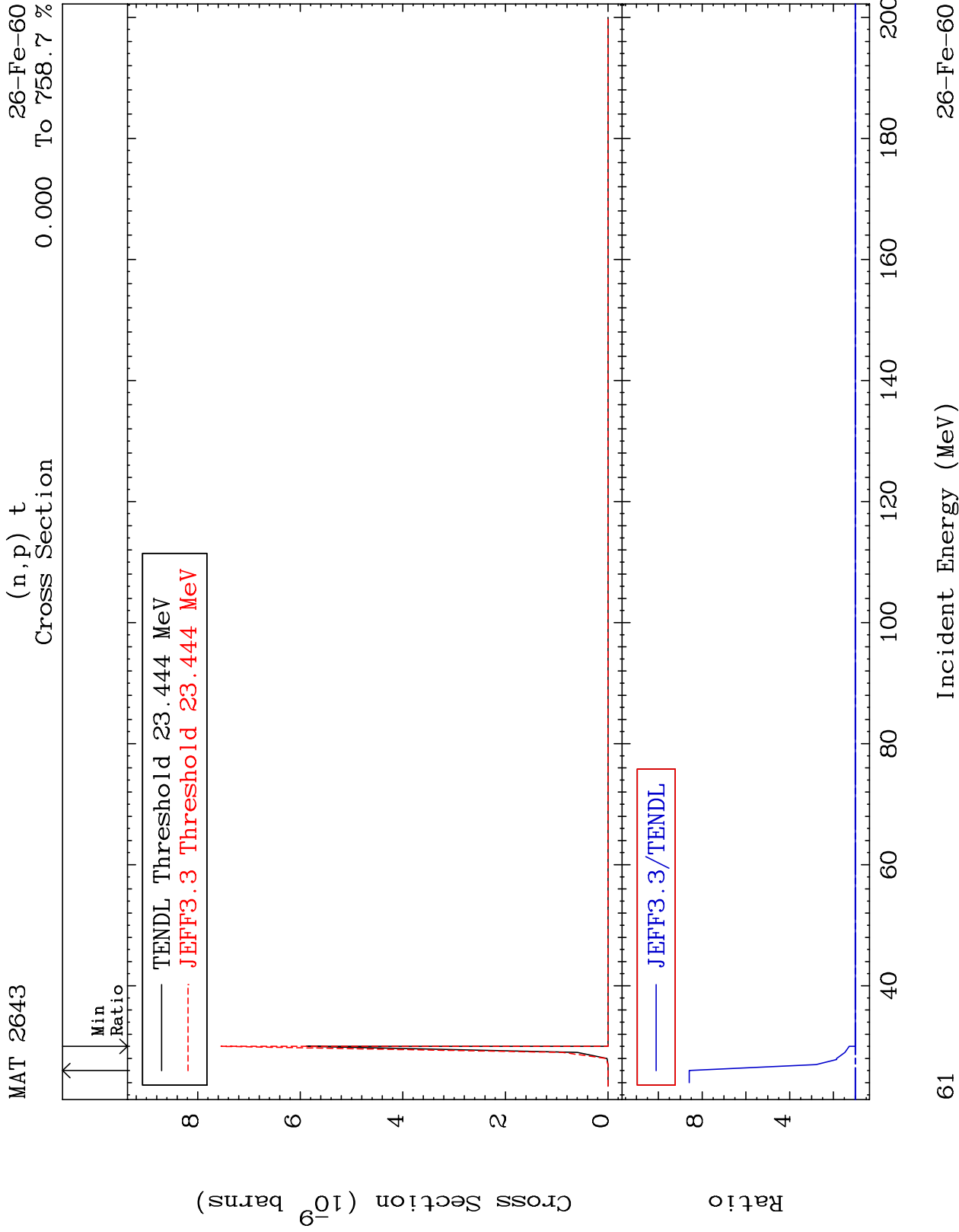
-59.32 To 9999. %

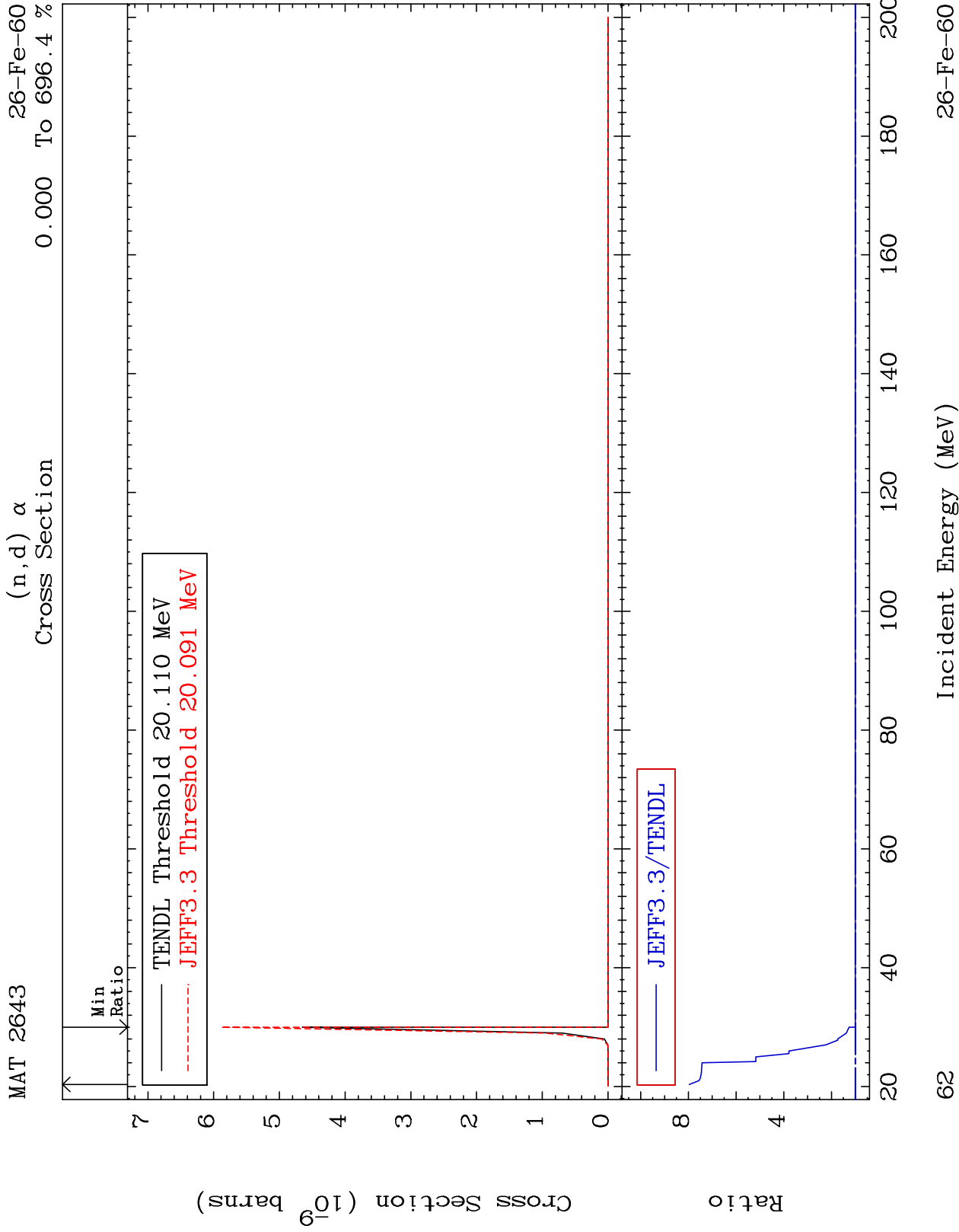


²⁶Fe-60

Incident Energy (MeV)

60

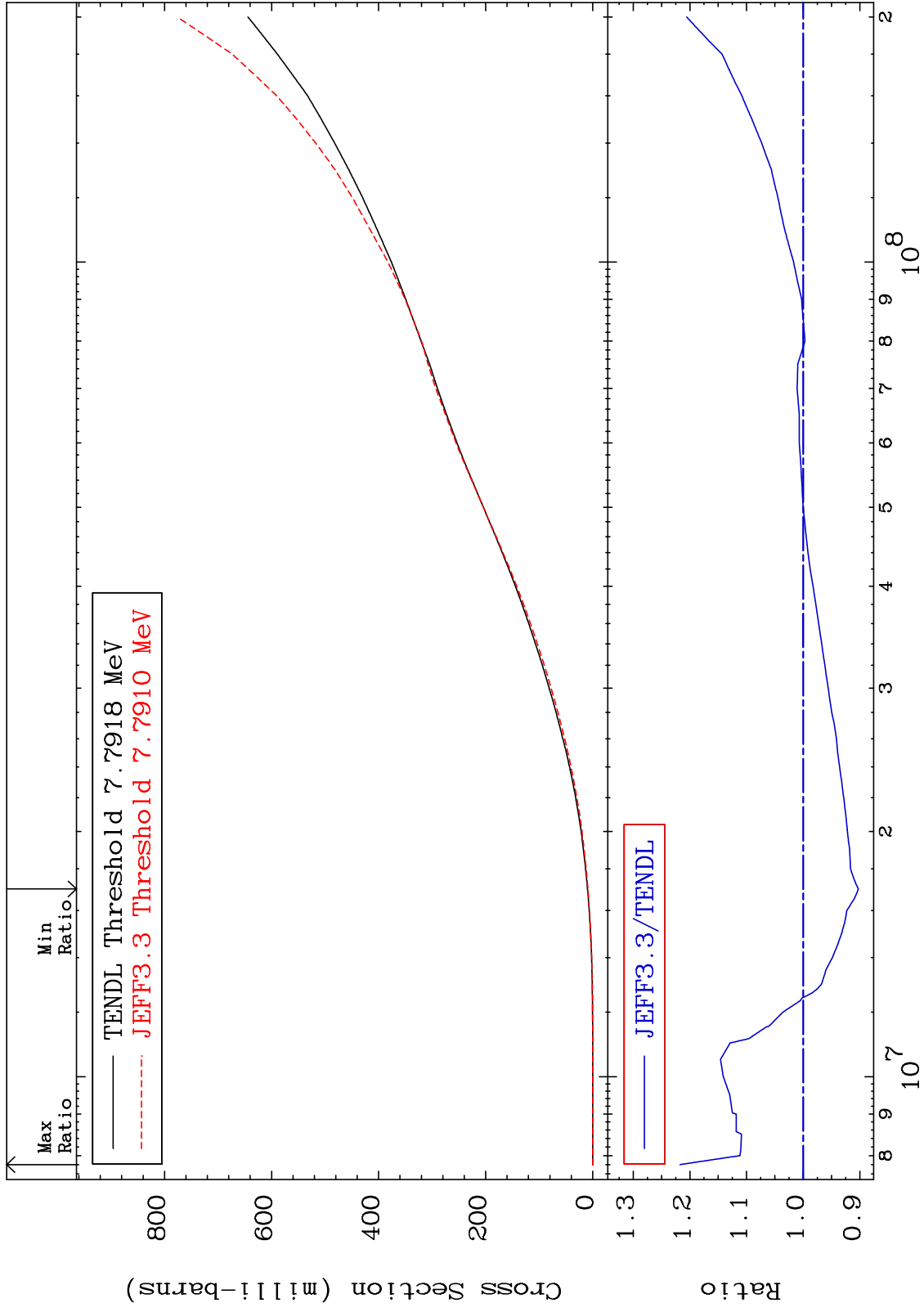




MAT 2643

Hydrogen Production
Cross Section

²⁶Fe-60
-9.730 To 21.74 %



63

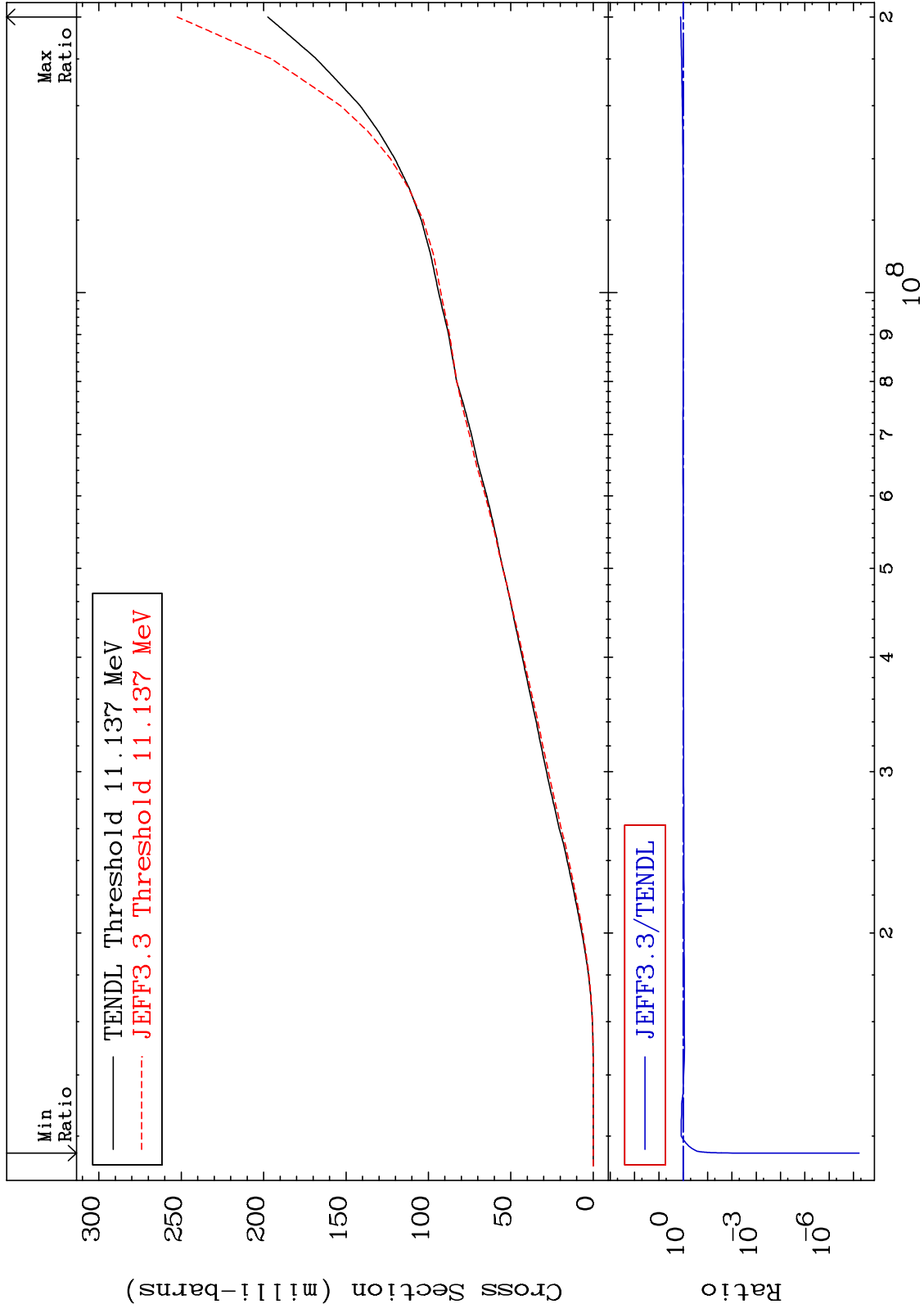
Incident Energy (eV)

²⁶Fe-60

MAT 2643

Deuterium Production
Cross Section

²⁶Fe-60
-100.0 To 27.81 %



64

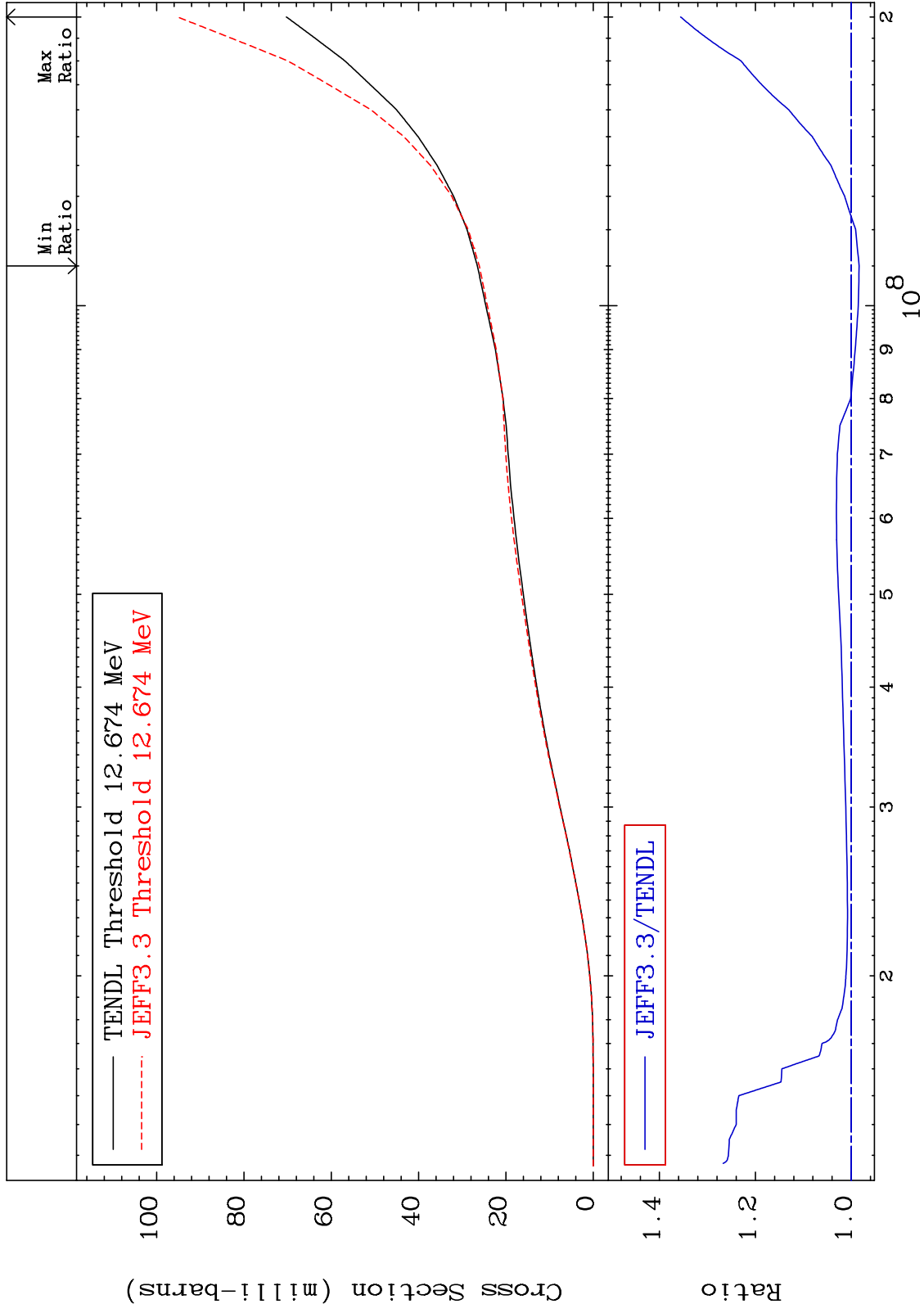
Incident Energy (eV)

²⁶Fe-60

MAT 2643

Tritium Production
Cross Section

$^{26}\text{Fe-60}$
-1.642 To 35.59 %



65

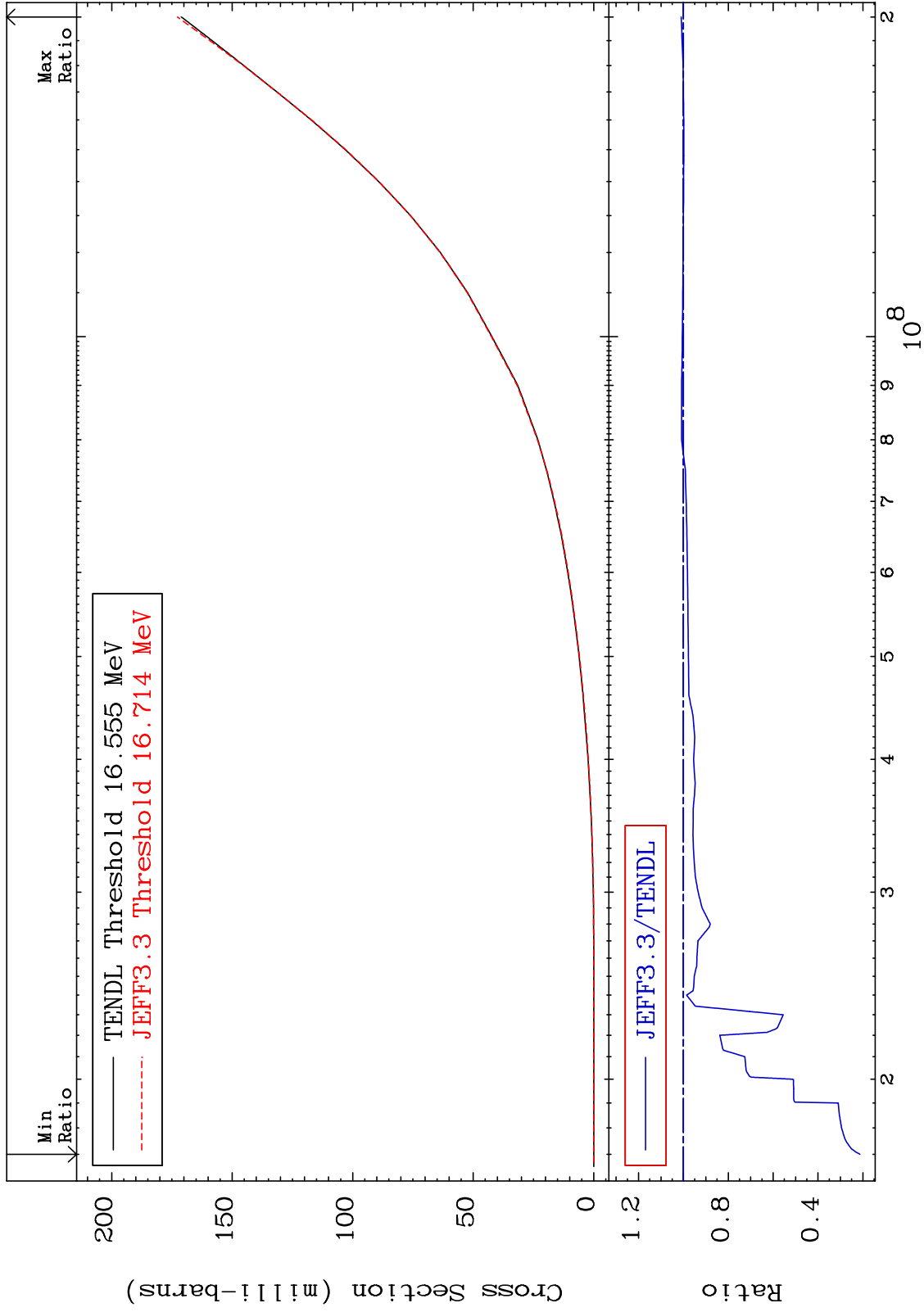
Incident Energy (eV)

$^{26}\text{Fe-60}$

MAT 2643

He-3 Production
Cross Section

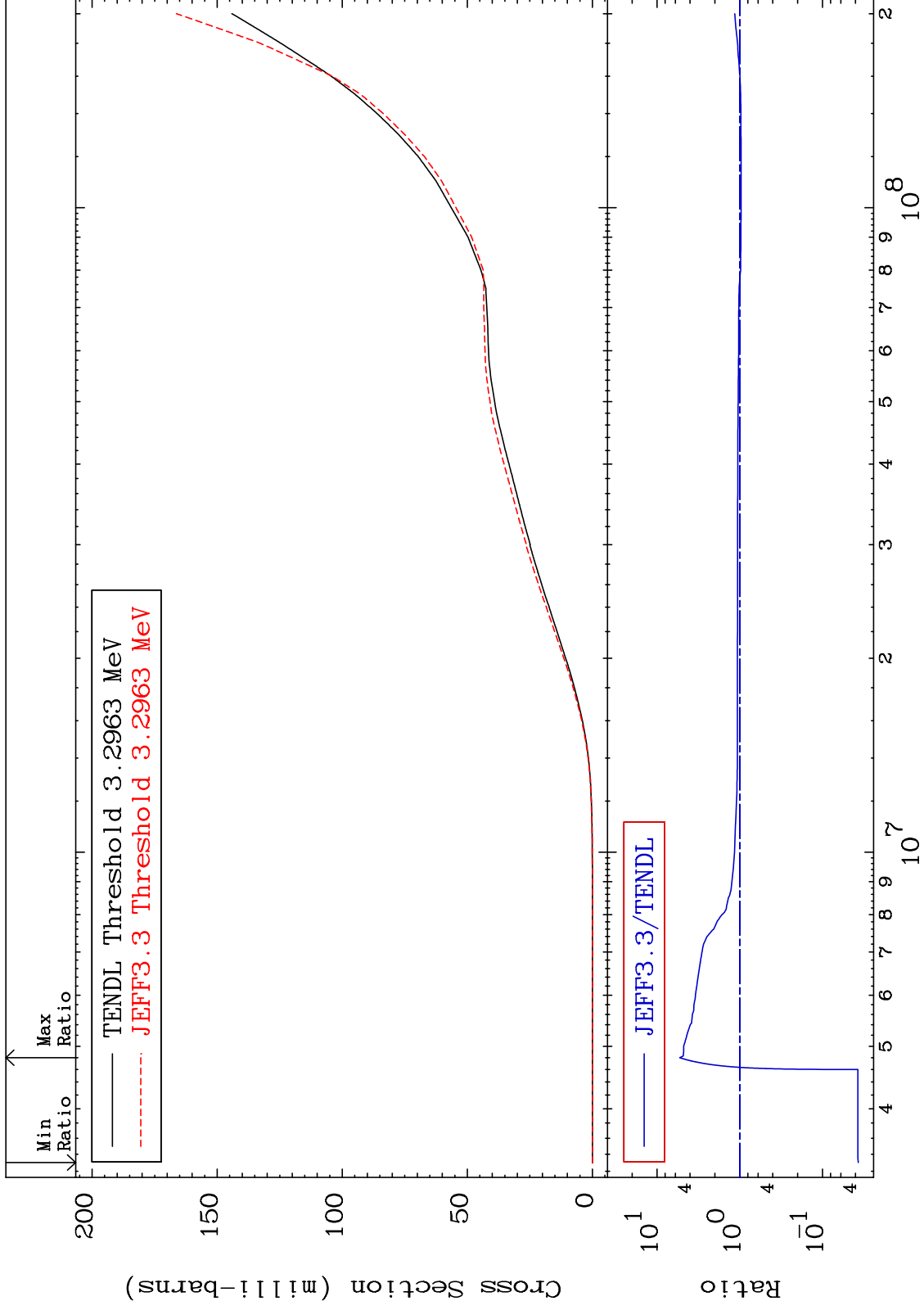
26-Fe-60
-78.70 To 0.966 %



MAT 2643

He-4 Production
Cross Section

²⁶Fe-60
-96.31 To 431.2 %



67

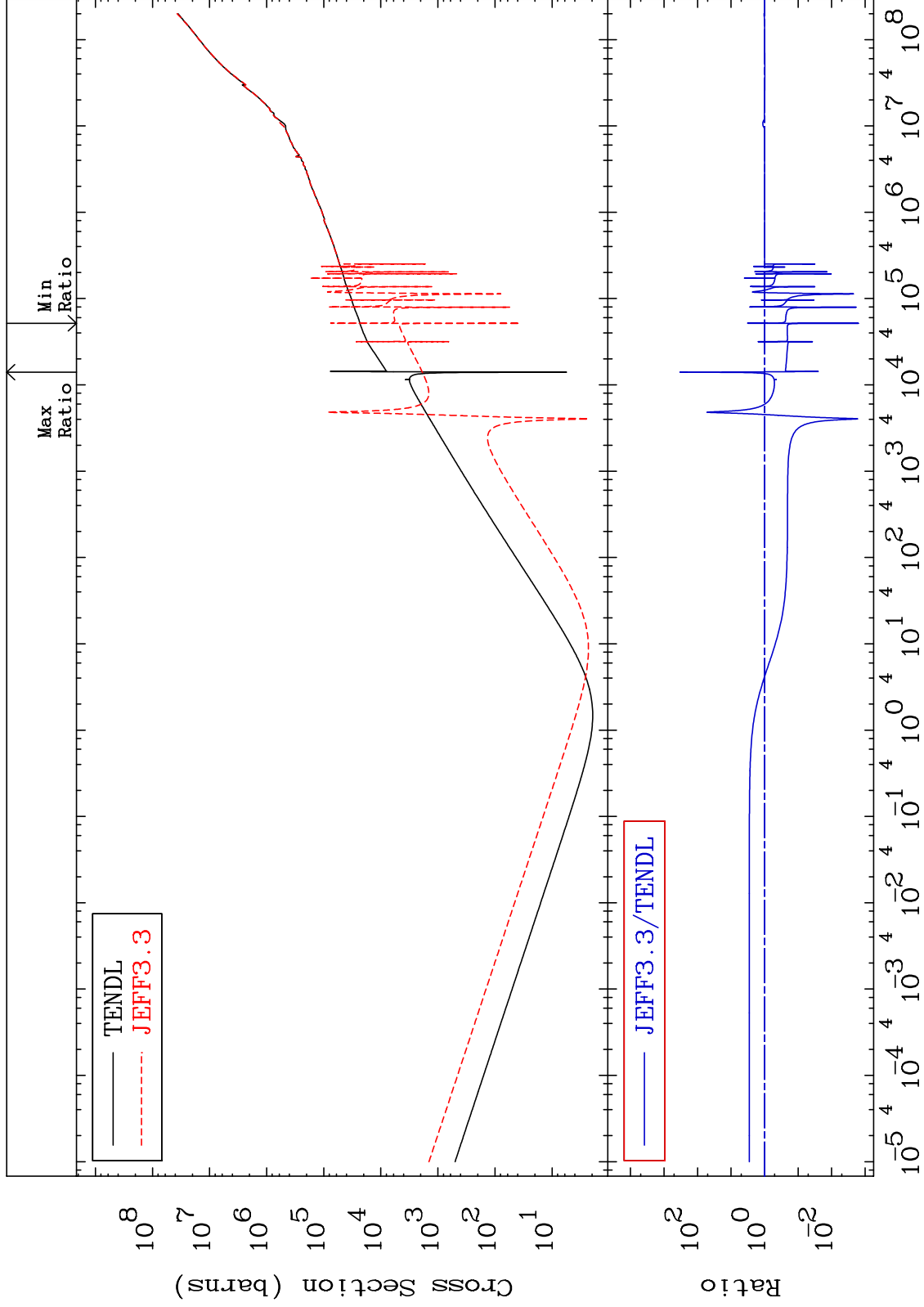
Incident Energy (eV)

²⁶Fe-60

MAT 2643

Kerma total (eV-barns)
Cross Section

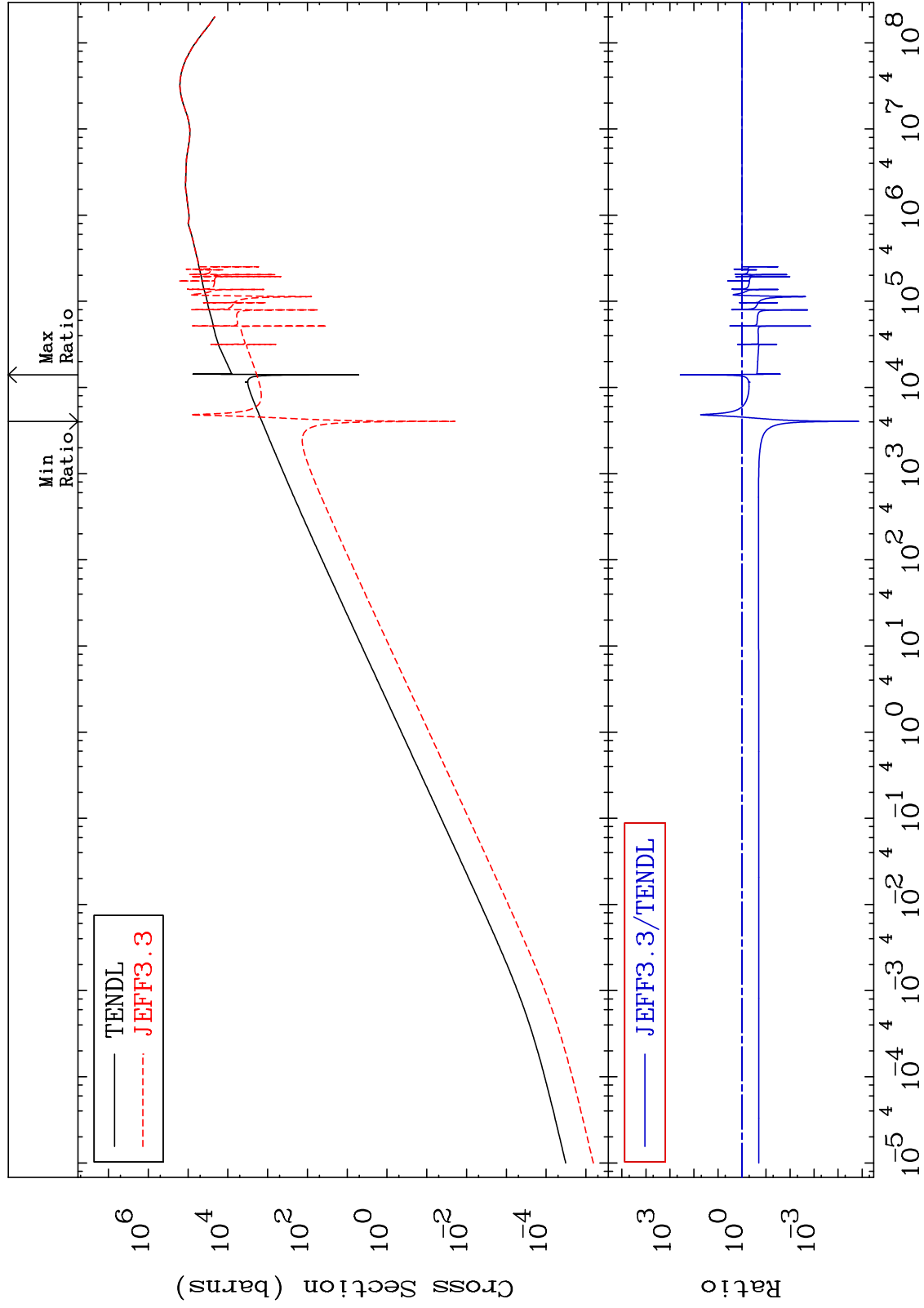
26-Fe-60
-99.84 To 9999. %



MAT 2643

Kerma elastic
Cross Section

26-Fe-60
-100.0 To 9999. %



69

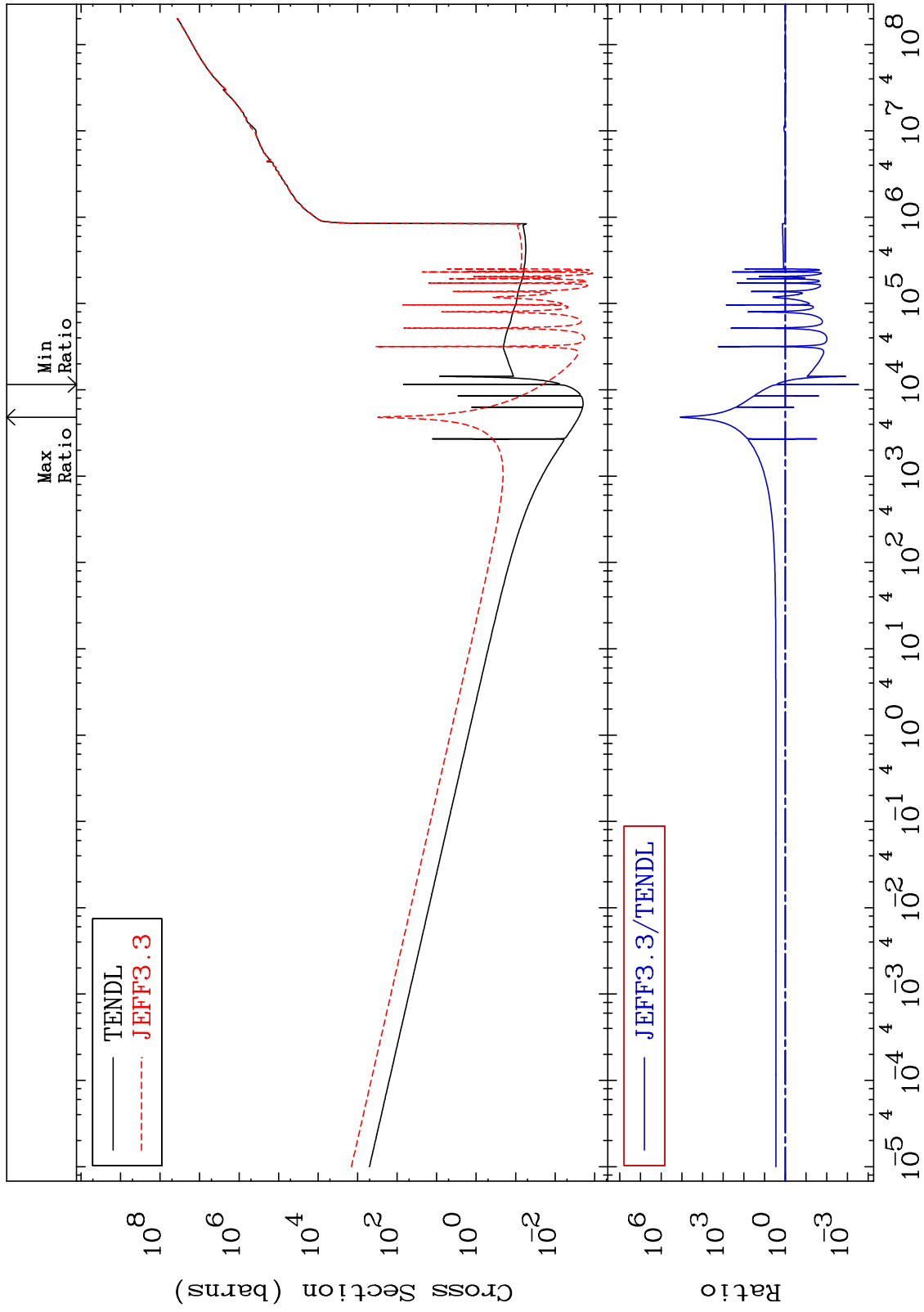
Incident Energy (eV)

26-Fe-60

MAT 2643

Kerma non-elastic (all but mt2)
Cross Section

26-Fe-60
-99.97 To 9999. %



70

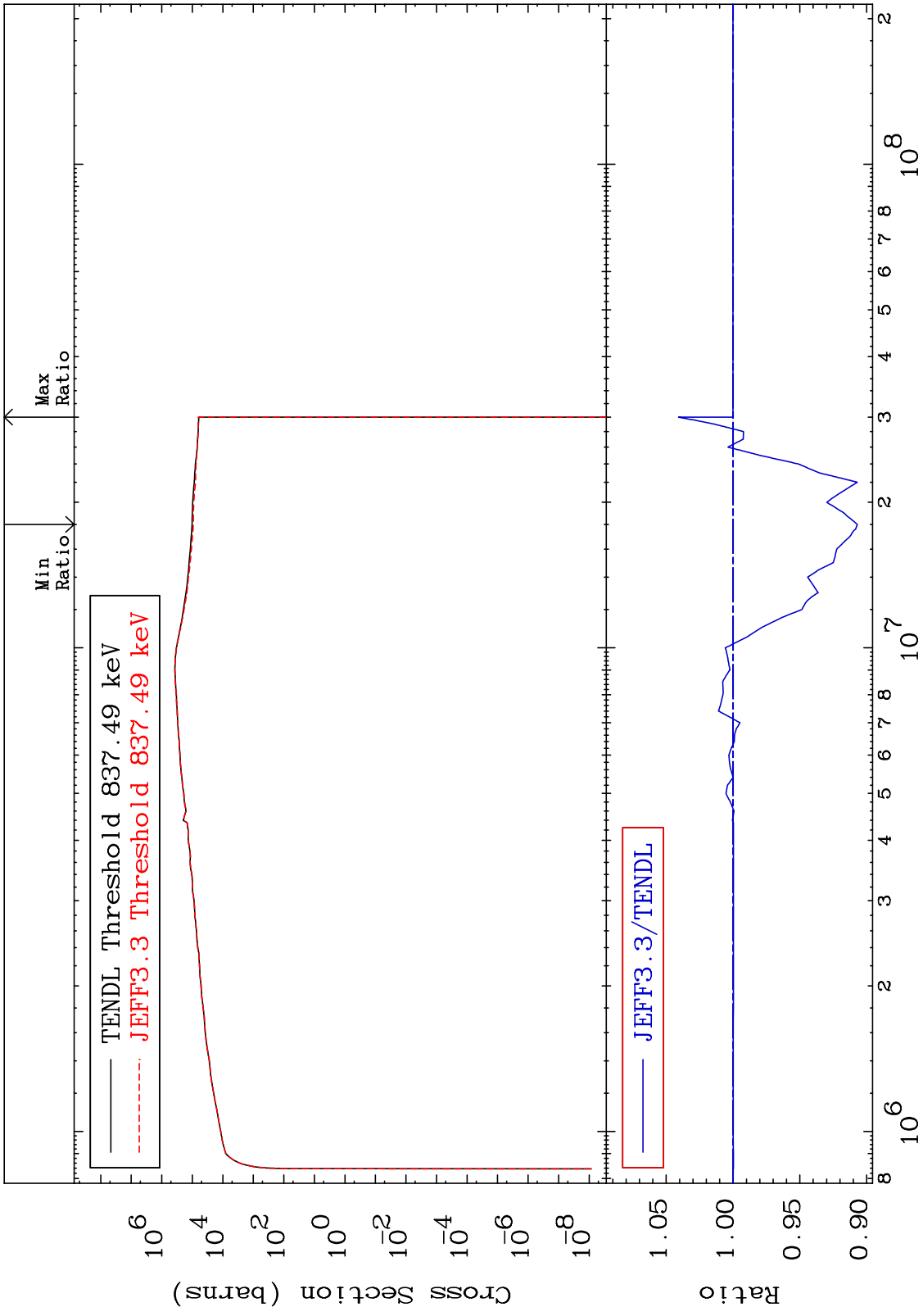
Incident Energy (eV)

26-Fe-60

MAT 2643

Kerma inelastic (mt51-91)
Cross Section

26-Fe-60
-9.298 To 4.070 %



71

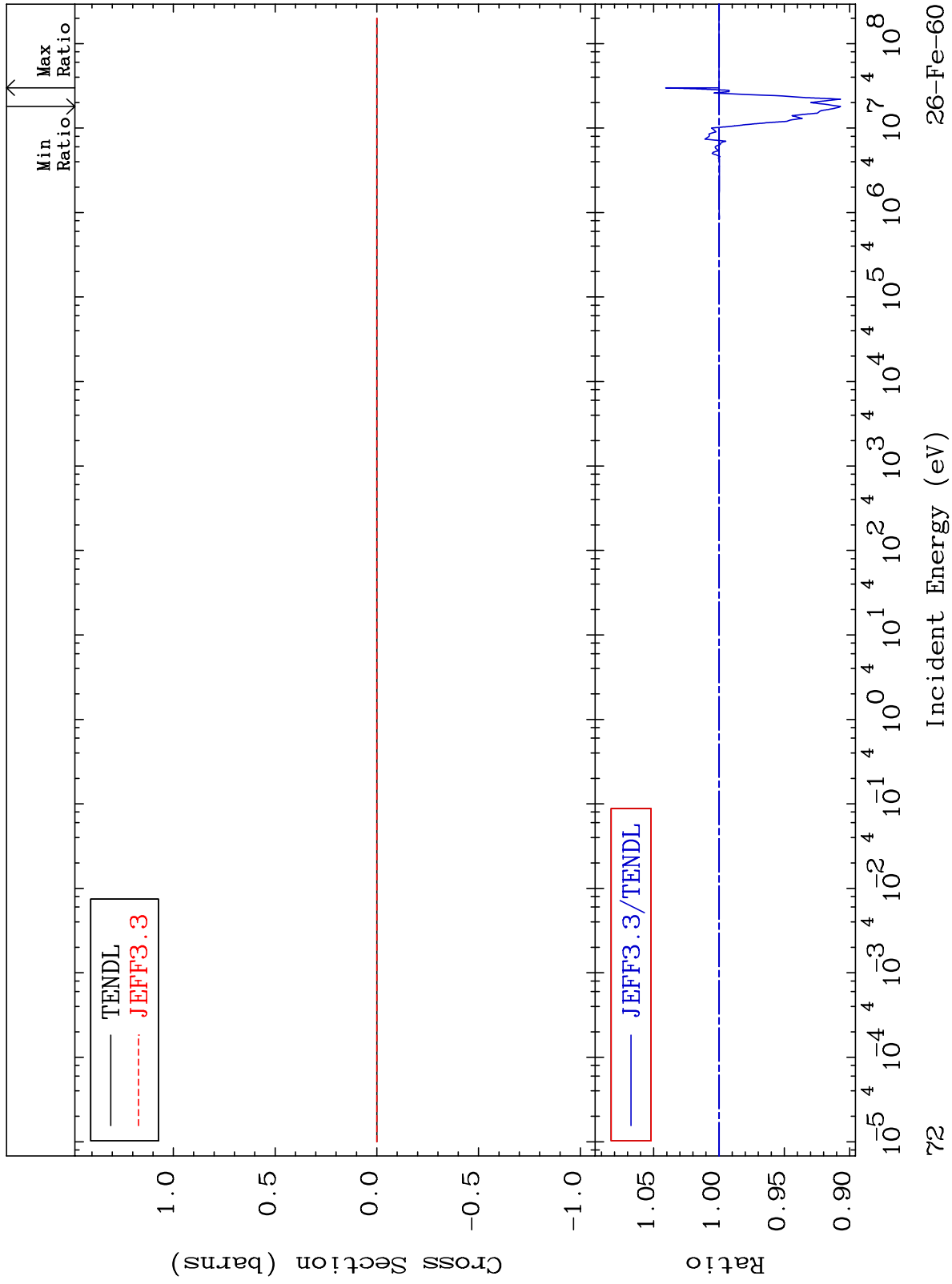
Incident Energy (eV)

26-Fe-60

MAT 2643

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

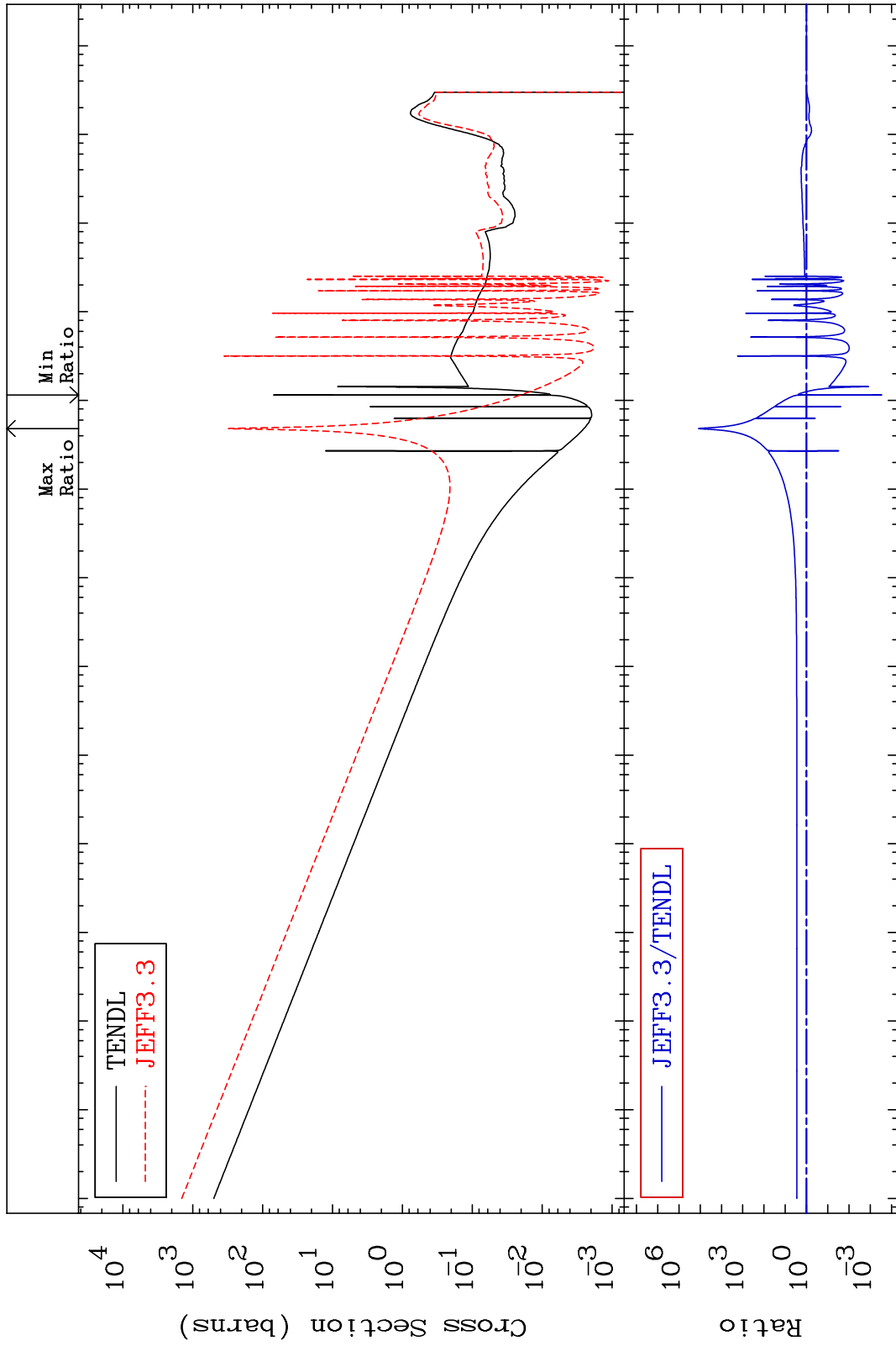
26-Fe-60
-9.298 To 4.070 %



MAT 2643

Kerma capture (mt102)
Cross Section

26-Fe-60
-99.97 To 9999. %



73

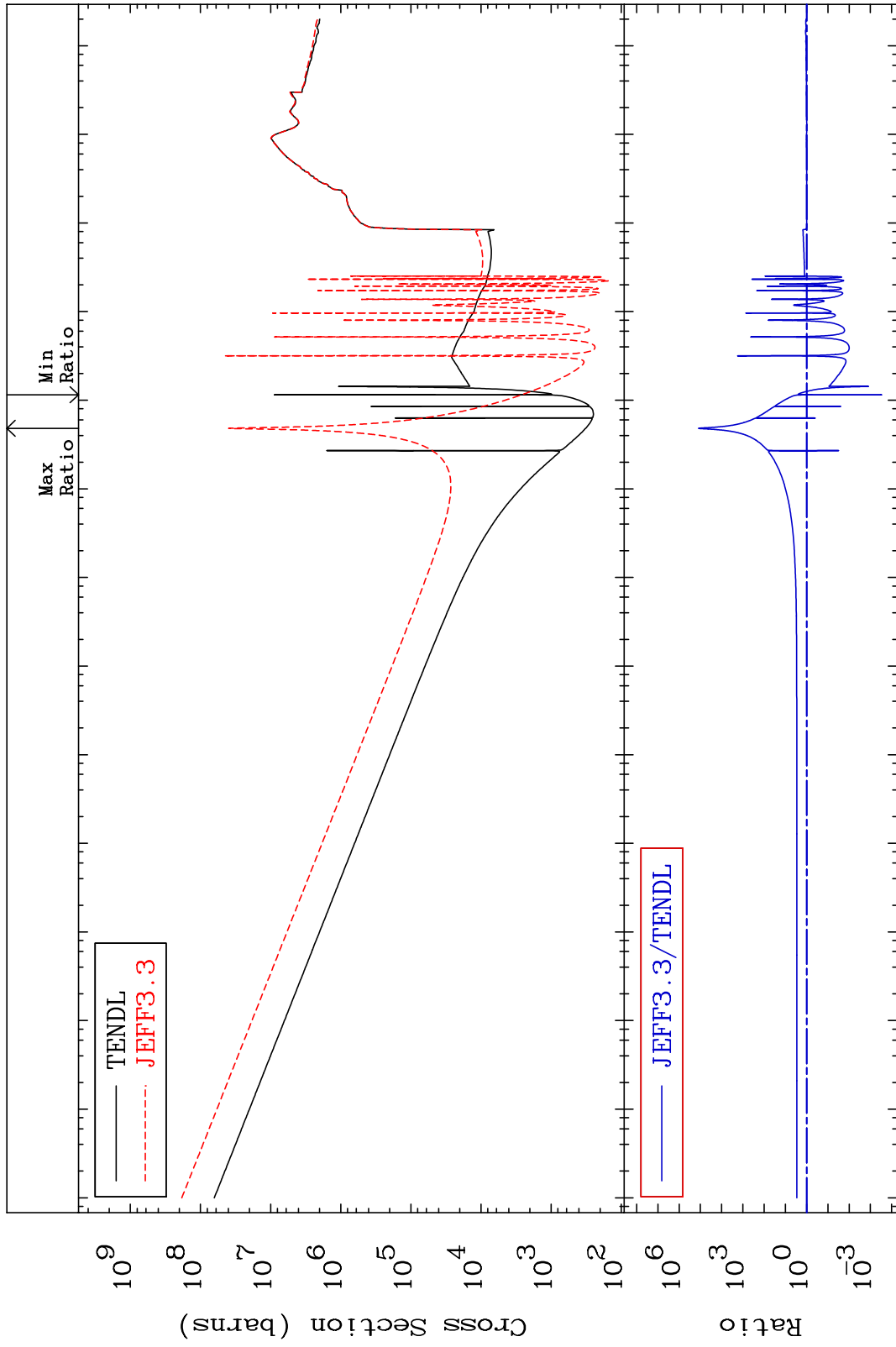
Incident Energy (eV)

26-Fe-60

MAT 2643

Total photon (eV-barns)
Cross Section

26-Fe-60
-99.97 To 9999. %



74

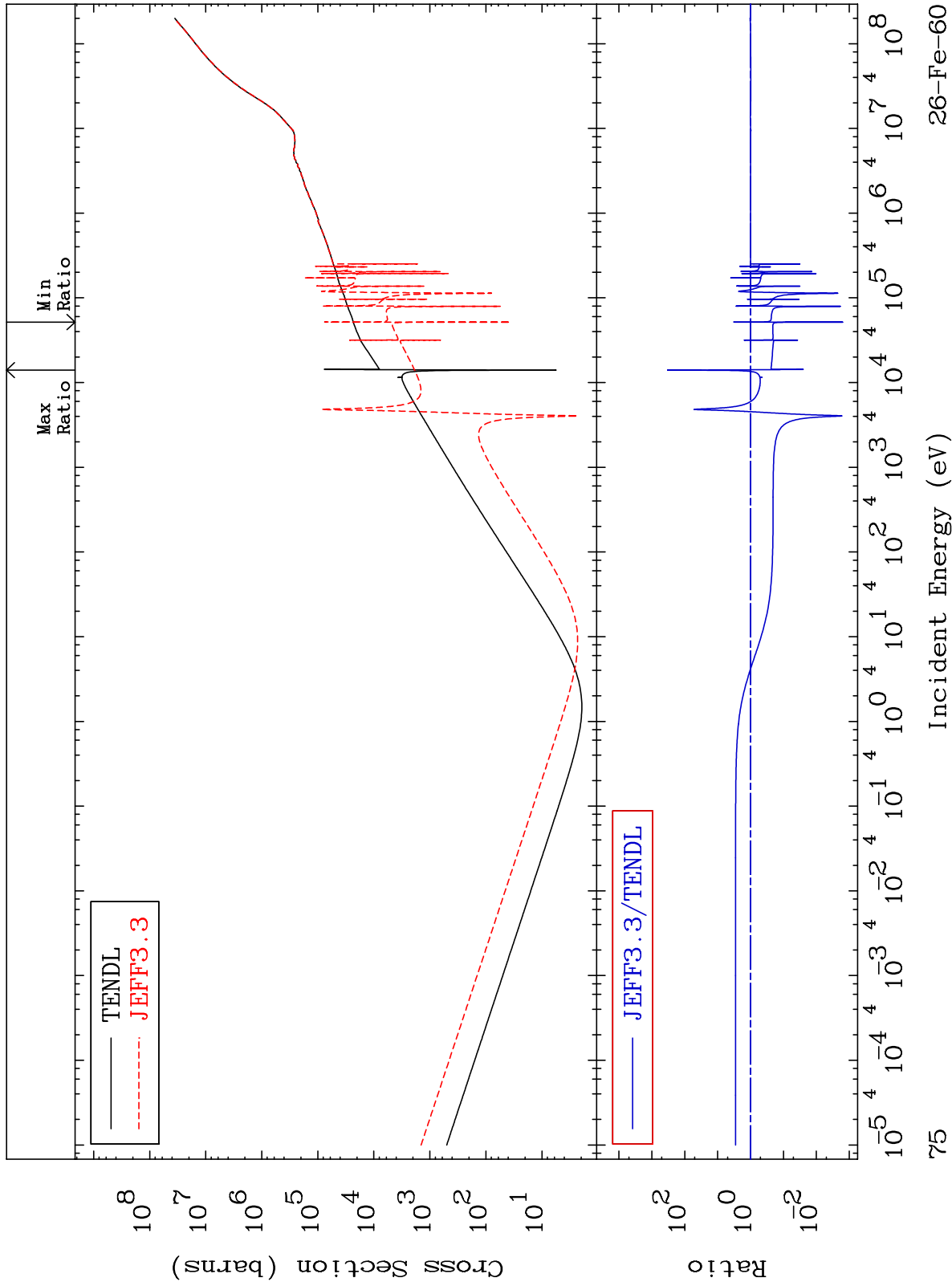
Incident Energy (eV)

26-Fe-60

MAT 2643

Total kinematic kerma (high limit)
Cross Section

26-Fe-60
-99.84 To 9999. %



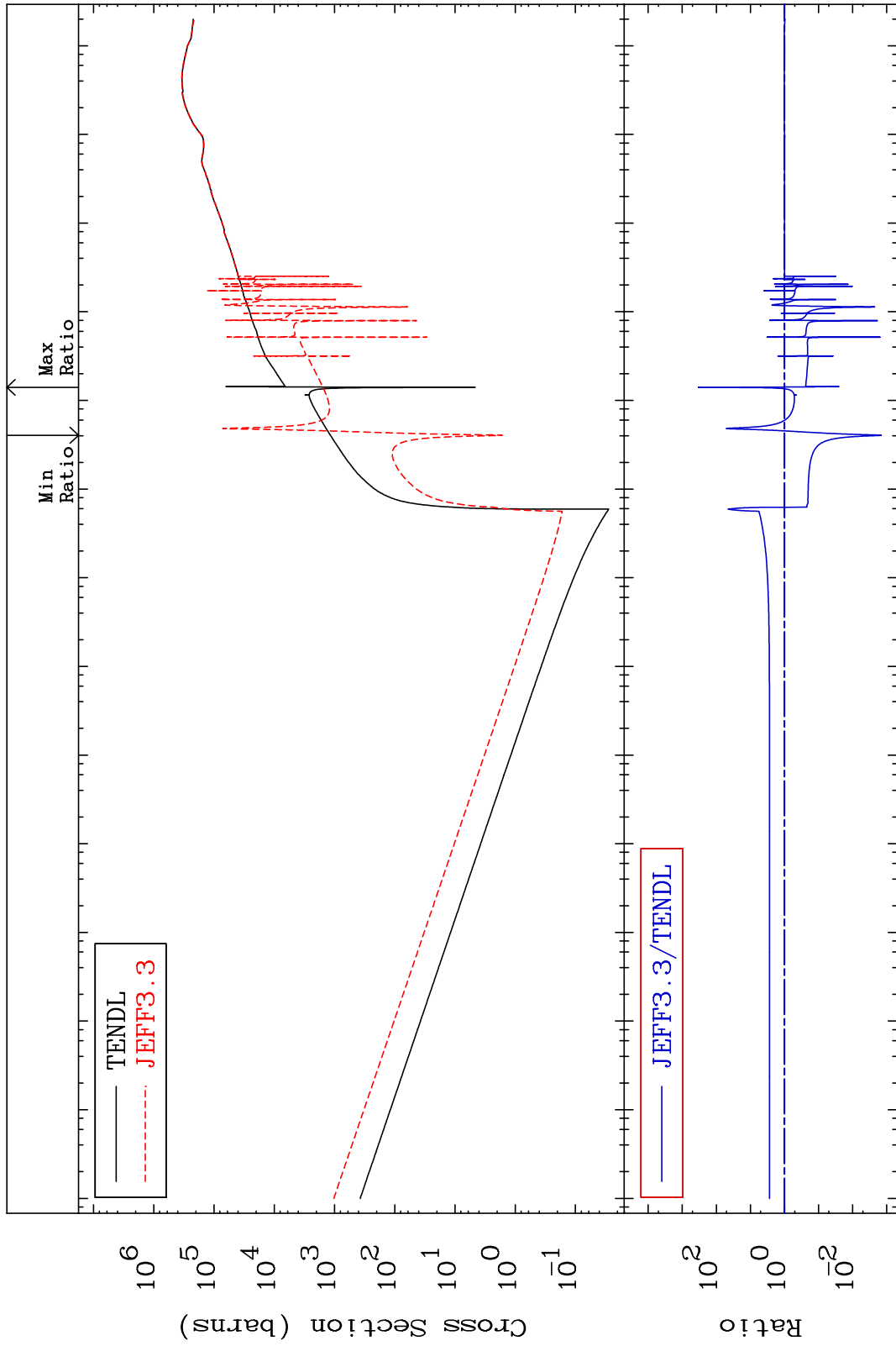
75

26-Fe-60

MAT 2643

Dpa total (eV-barns)
Cross Section

26-Fe-60
-99.86 To 9999. %



76

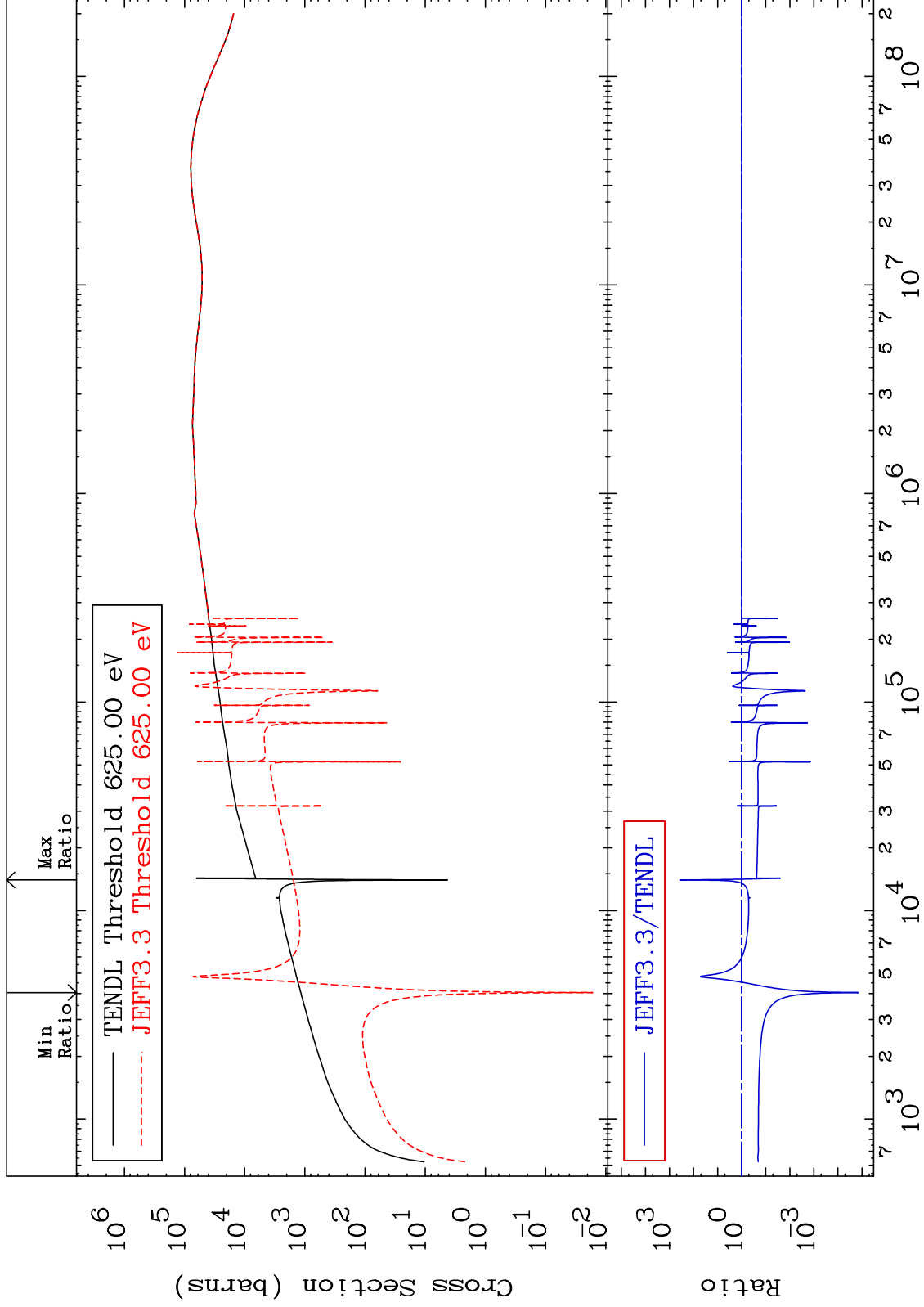
Incident Energy (eV)

26-Fe-60

MAT 2643

Dpa elastic (mt2)
Cross Section

26-Fe-60
-100.0 To 9999. %



77

Incident Energy (eV)

26-Fe-60

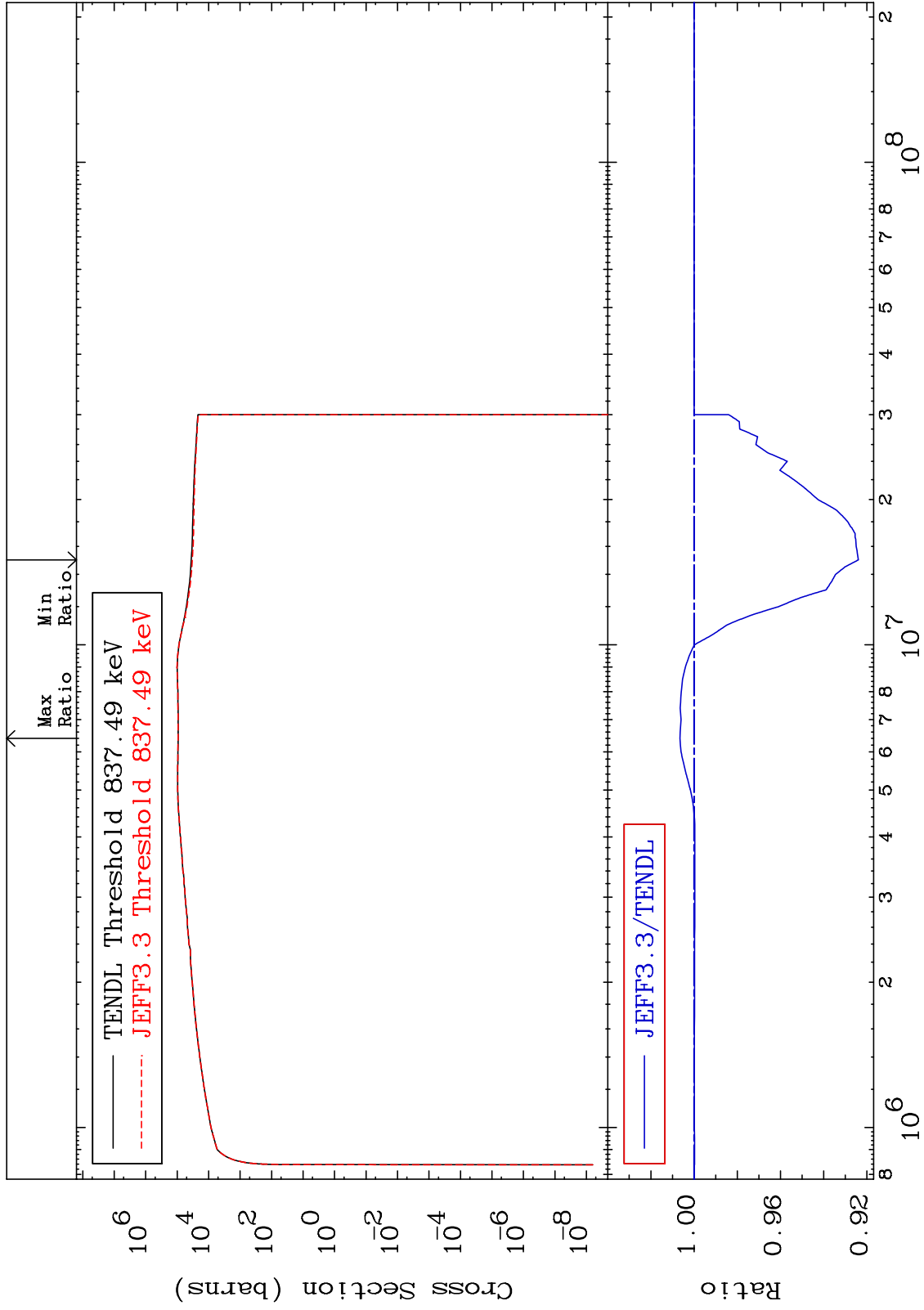
MAT 2643

Dpa inelastic (mt51-91)

²⁶Fe-60

-7.604 To 0.650 %

Cross Section



78

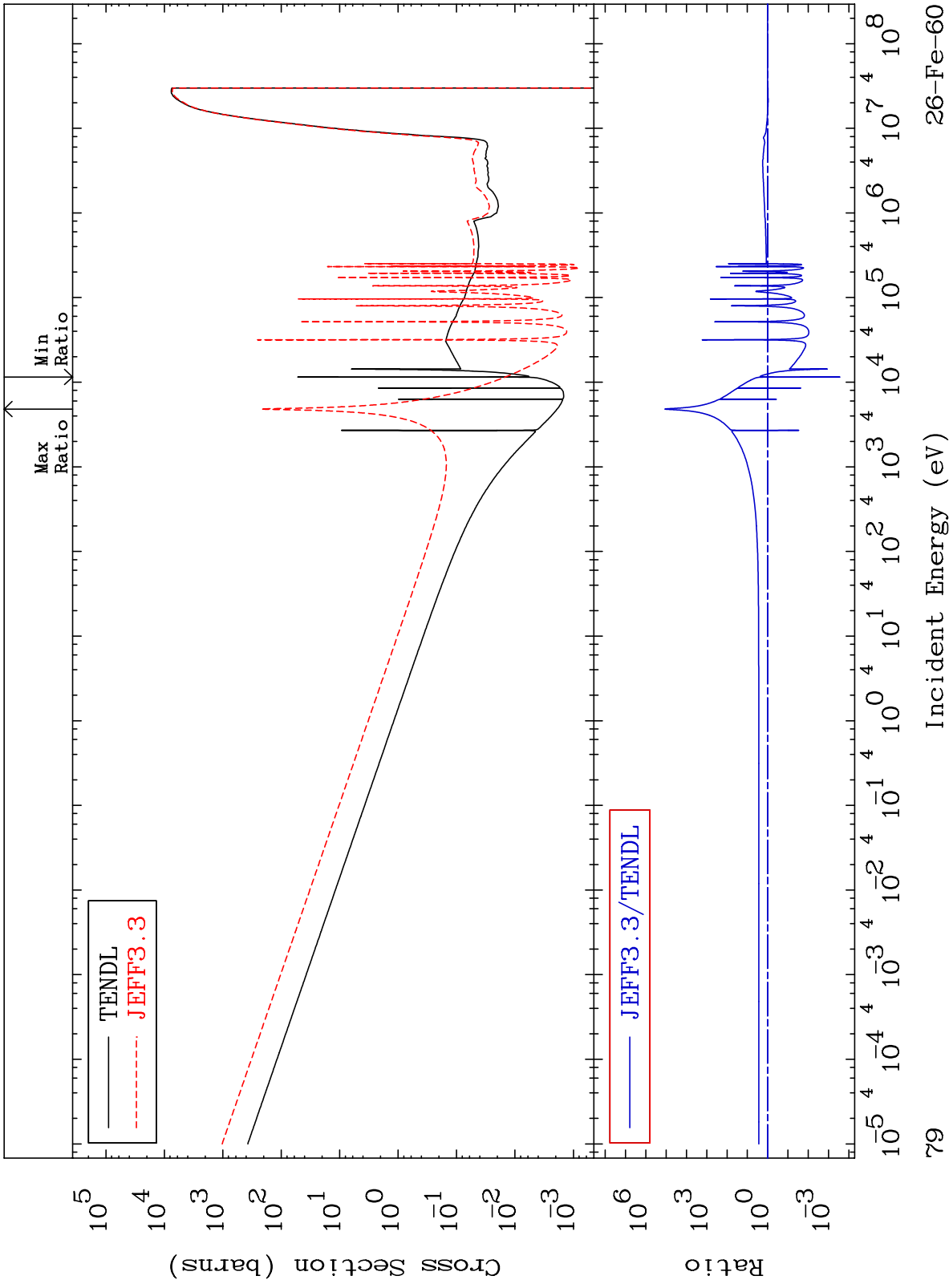
Incident Energy (eV)

²⁶Fe-60

MAT 2643

Dpa disappearance (mt102 -120)
Cross Section

26-Fe-60
-99.97 To 9999. %



79

Incident Energy (eV)

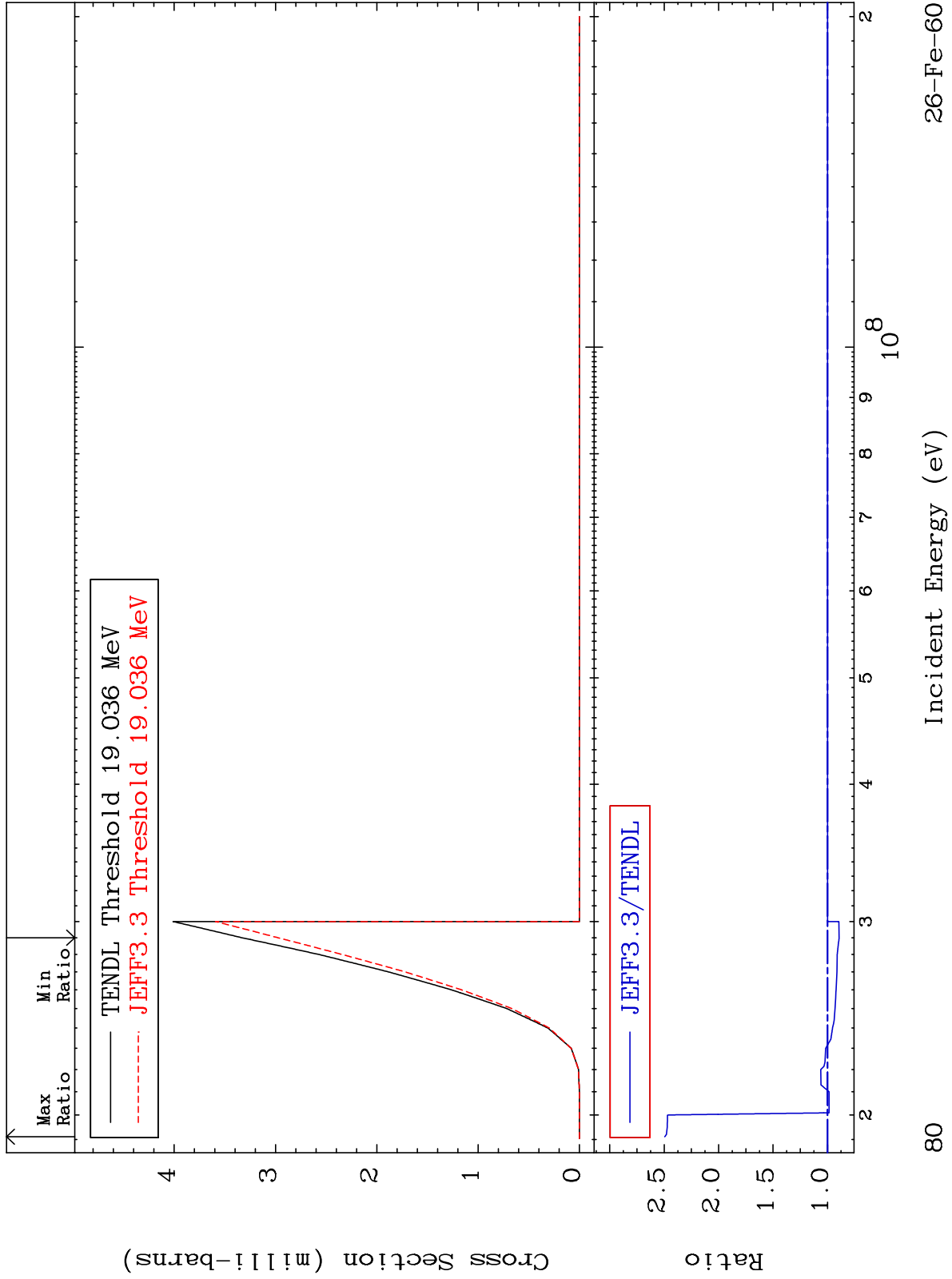
26-Fe-60

MAT 2643

(n, n') d:25-Mn-58g

26-Fe-60

Radionuclide Production Cross Section -10.69 To 149.5 %



80

Incident Energy (eV)

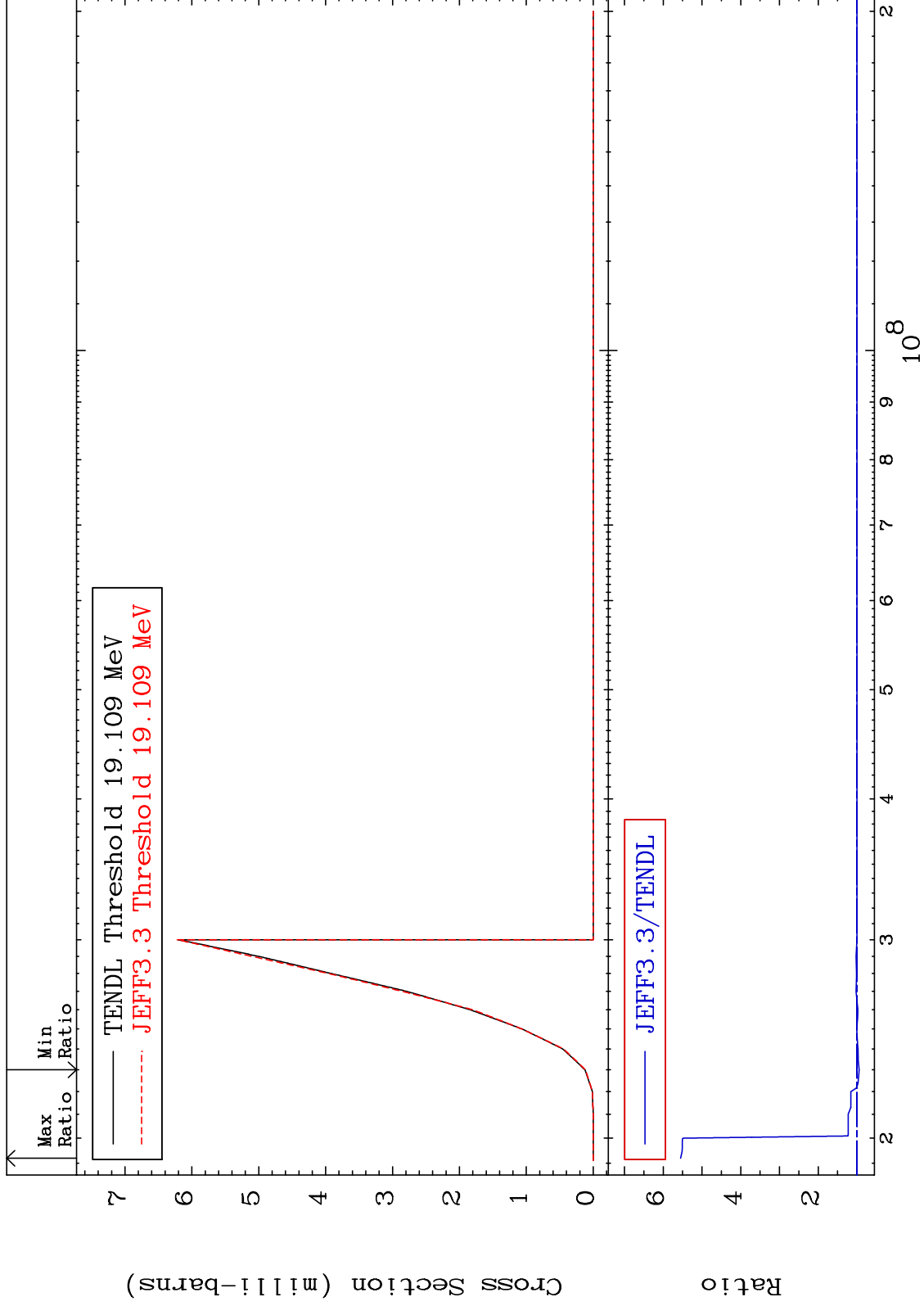
26-Fe-60

MAT 2643

(n, n') d:25-Mn-58m1

26-Fe-60

Radionuclide Production Cross Section -5.749 To 455.1 %

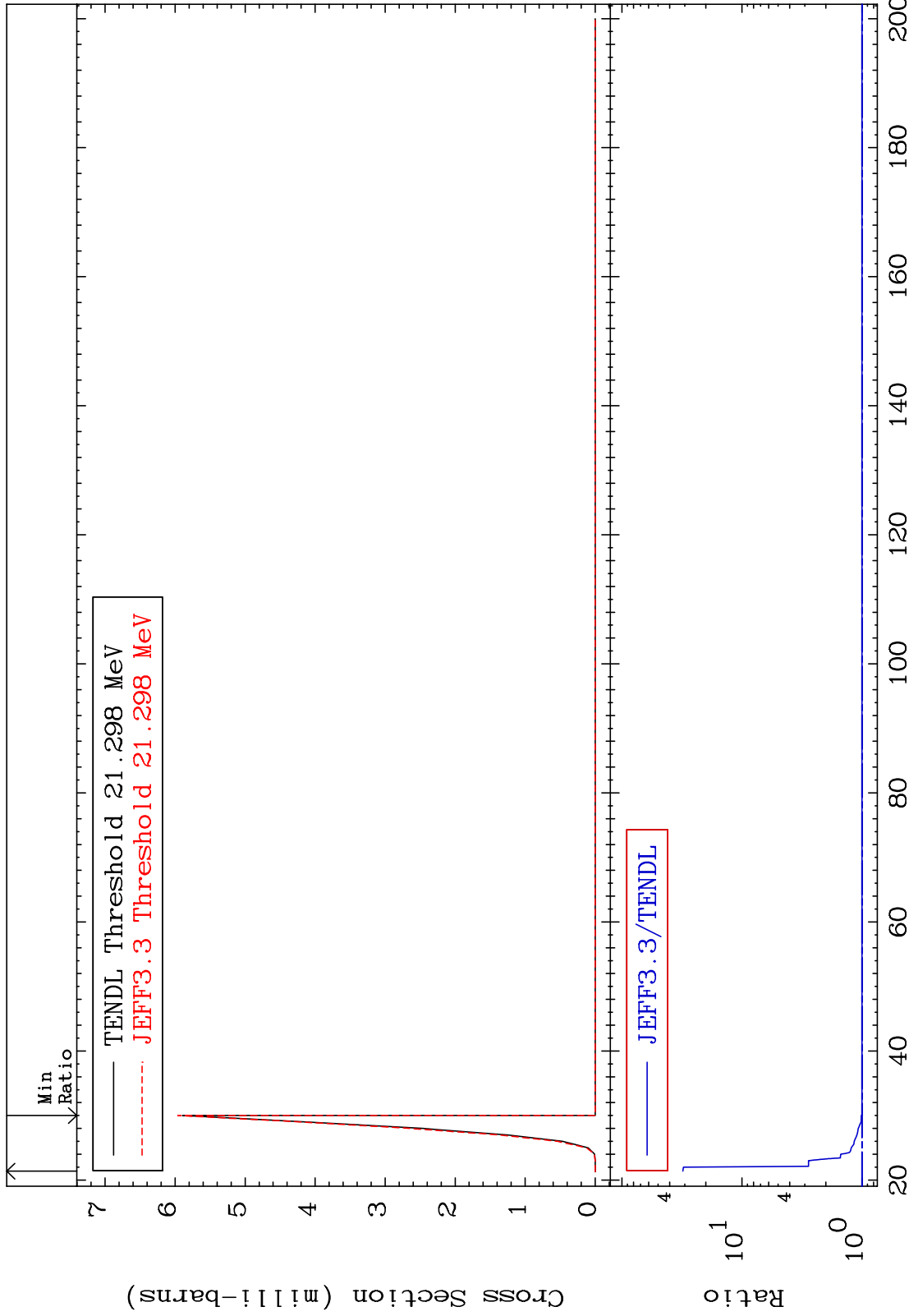


MAT 2643

(n,2n) p:25-Mn-58g

26-Fe-60

Radionuclide Production Cross Section 0.000 To 3010. %



82

Incident Energy (MeV)

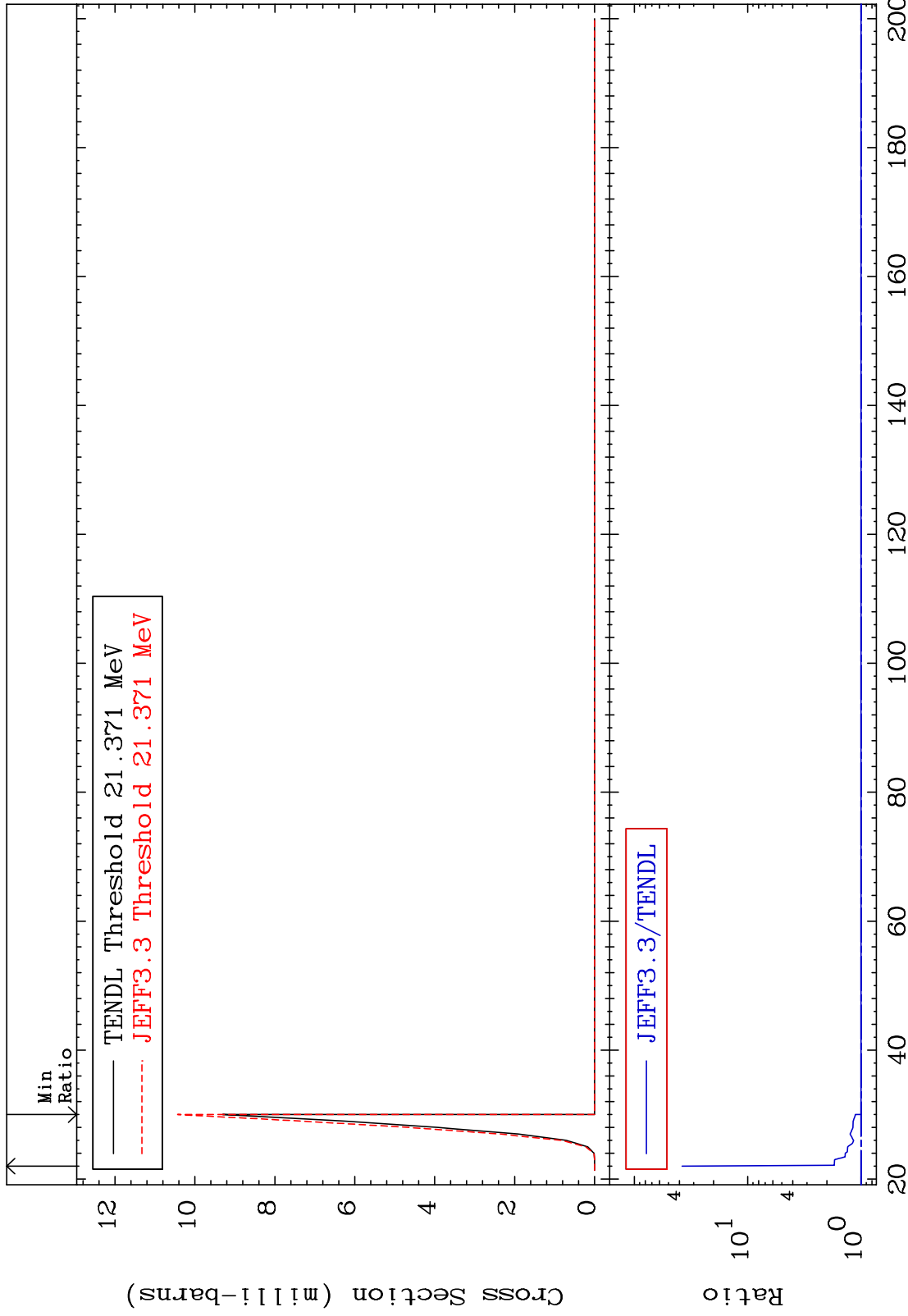
26-Fe-60

MAT 2643

(n,2n) p:25-Mn-58m1

26-Fe-60

Radionuclide Production Cross Section 0.000 To 3692. %



83

Incident Energy (MeV)

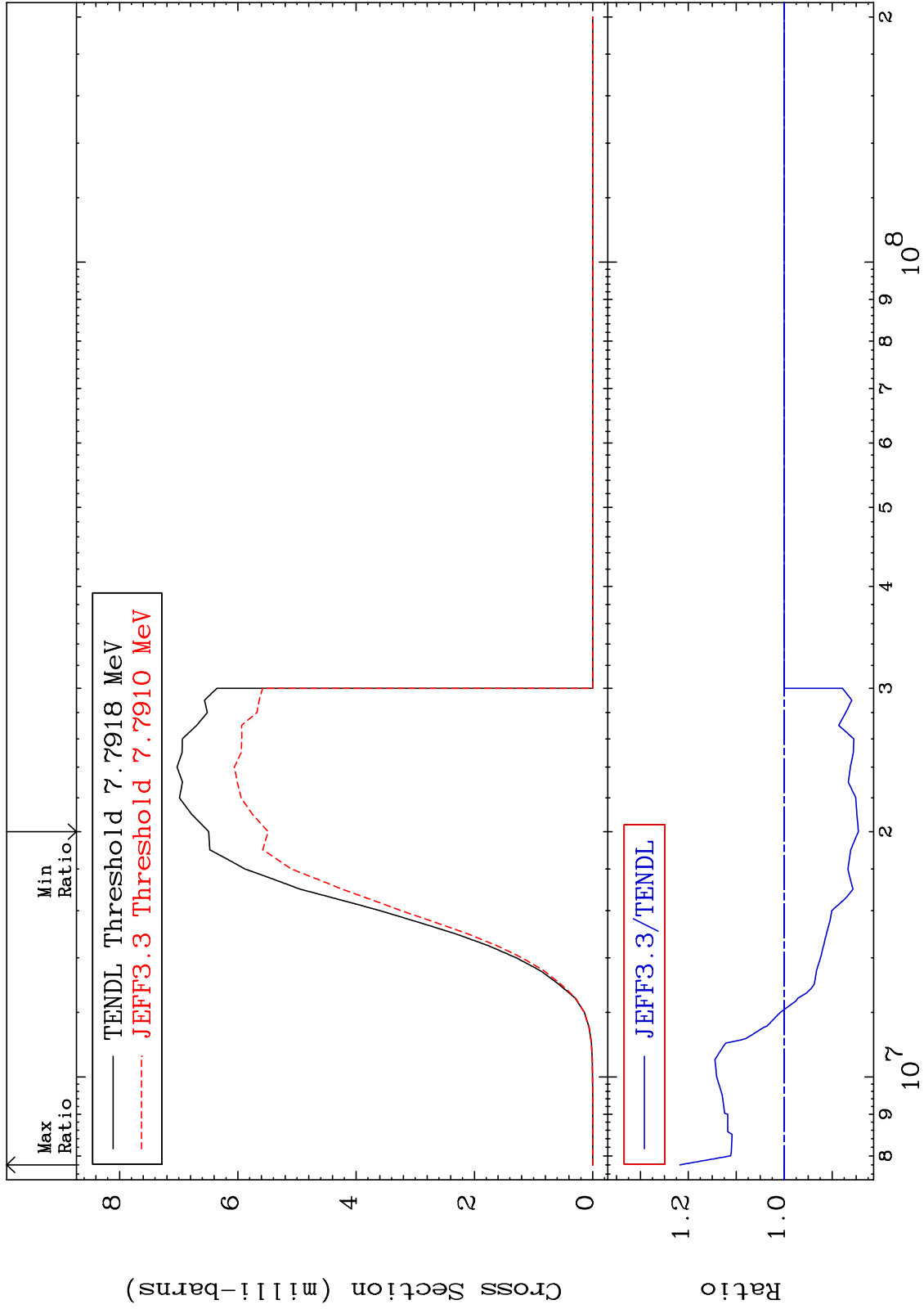
26-Fe-60

MAT 2643

(n, p) : 25-Mn-60g

26-Fe-60

Radionuclide Production Cross Section -15.47 To 21.74 %

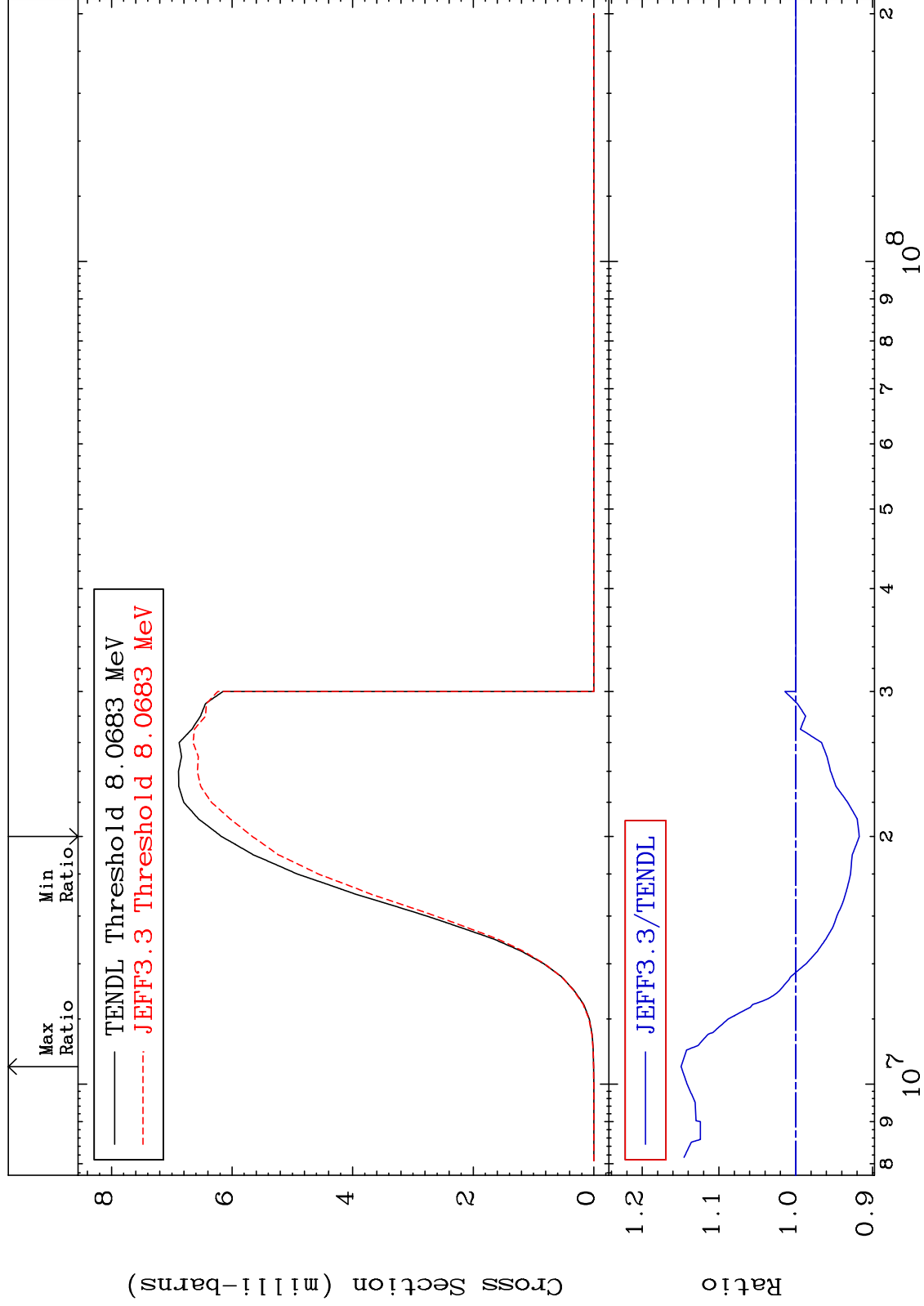


MAT 2643

(n, p):25-Mn-60m1

26-Fe-60

Radionuclide Production Cross Section -8.283 To 14.94 %



85

Incident Energy (eV)

26-Fe-60

MAT 2643

(n, t) : 25-Mn-58g

26-Fe-60

Radionuclide Production Cross Section -9.498 To 27.08 %

