

Program EVALPLOT
(Version 2018-1)

by

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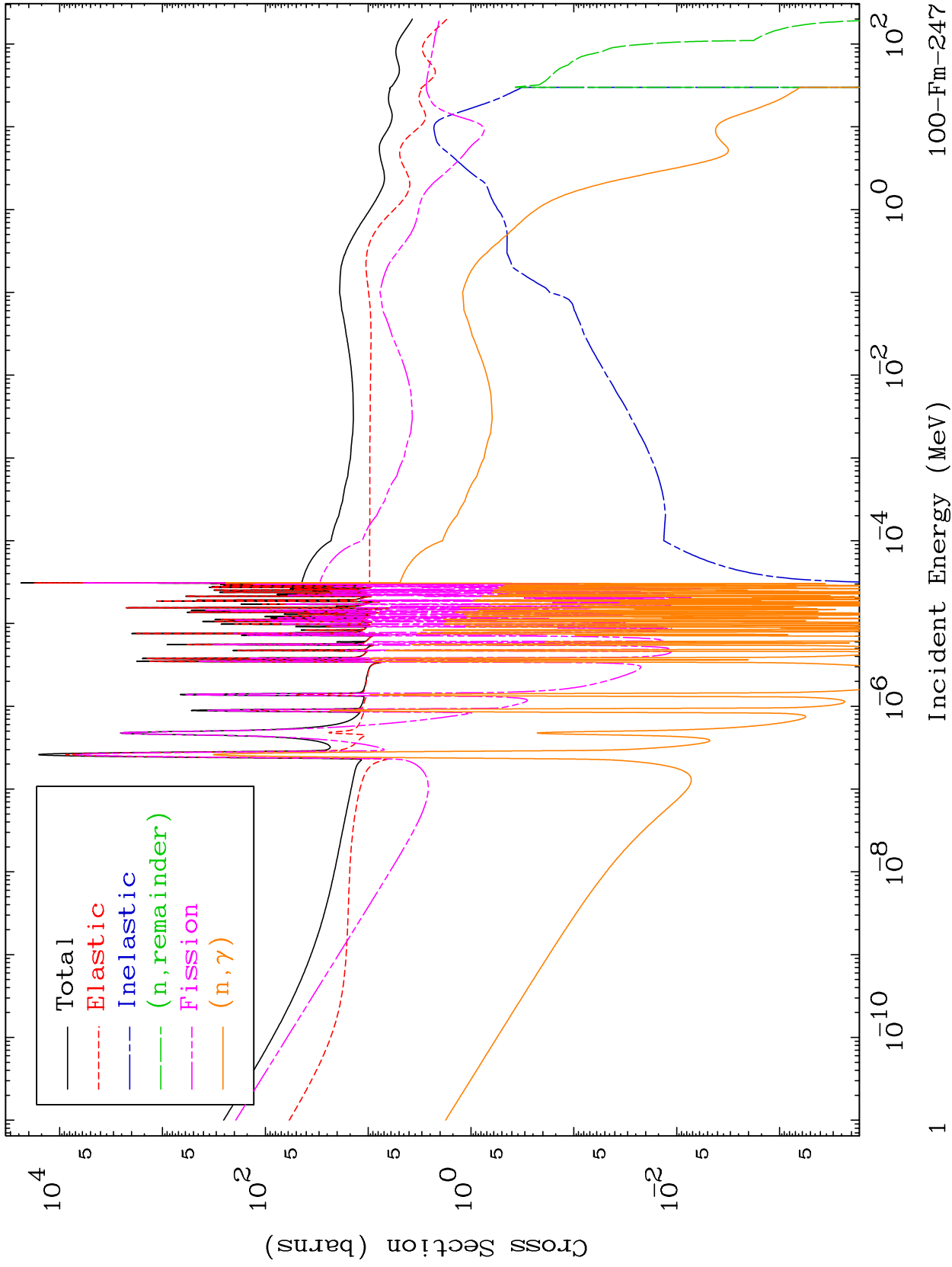
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9933

Major
293 Kelvin Cross Sections

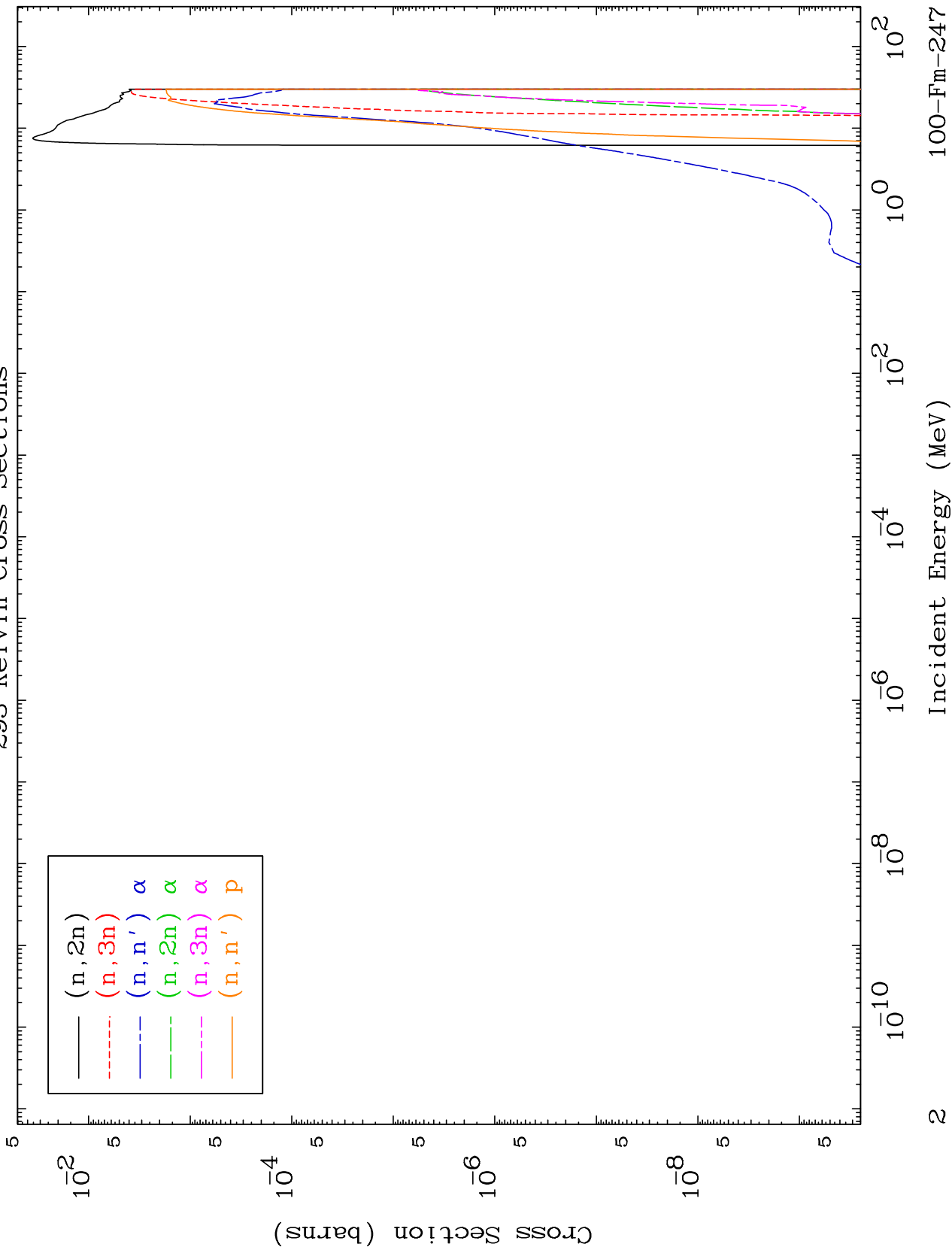
100-Fm-247

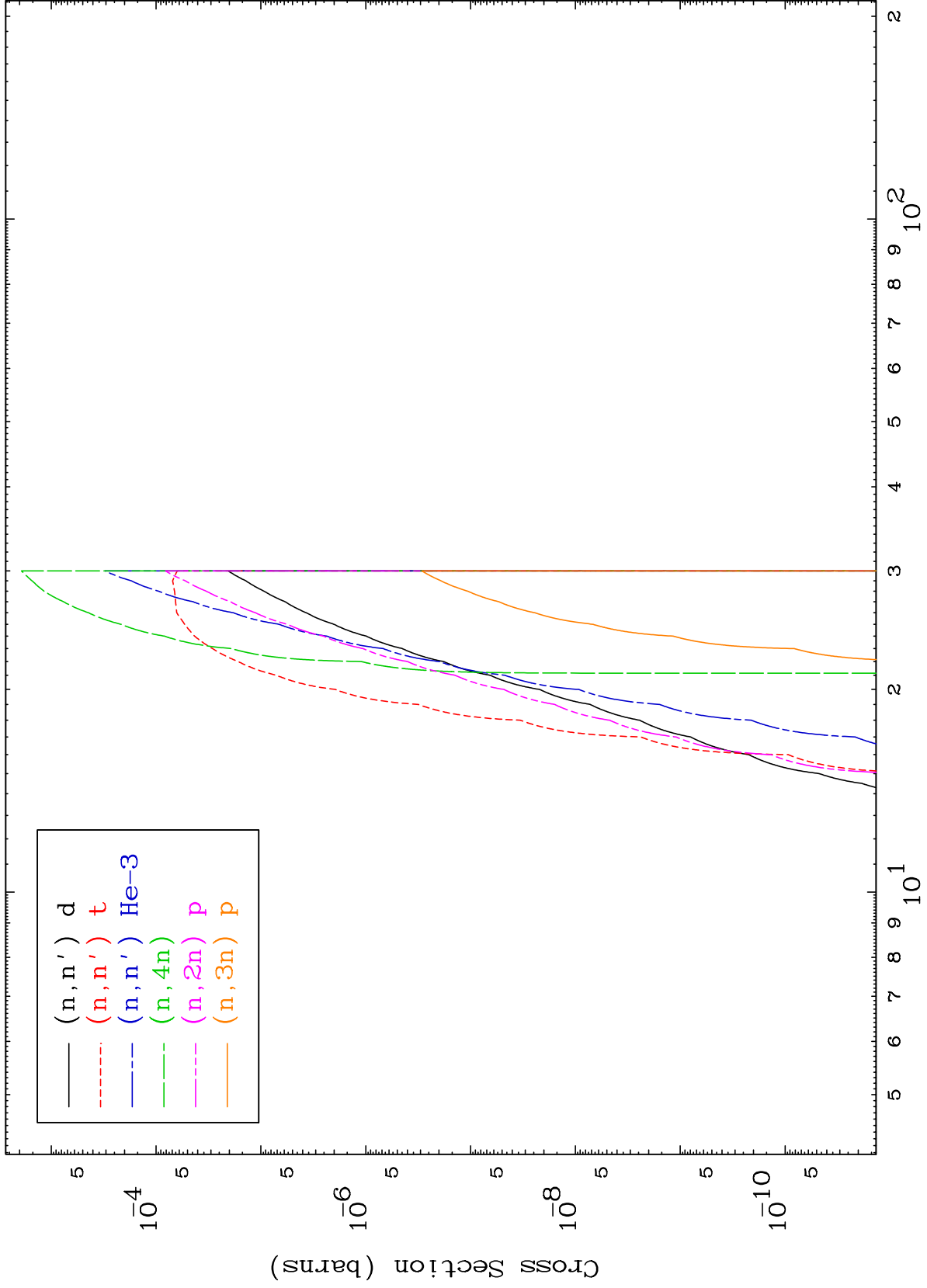


MAT 9933

Neutron Production
293 Kelvin Cross Sections

100-Fm-247

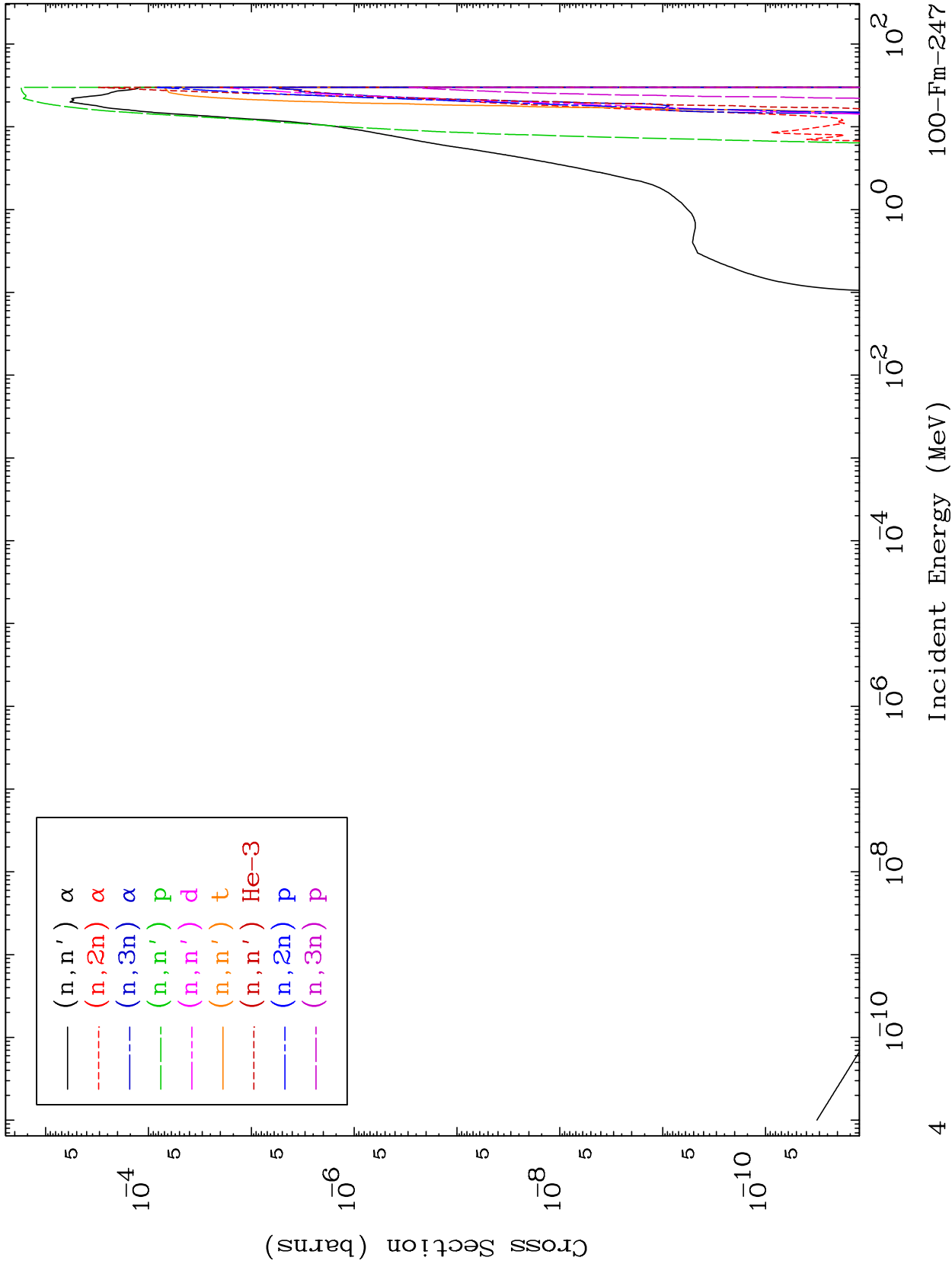




MAT 9933

Charged Particle
293 Kelvin Cross Sections

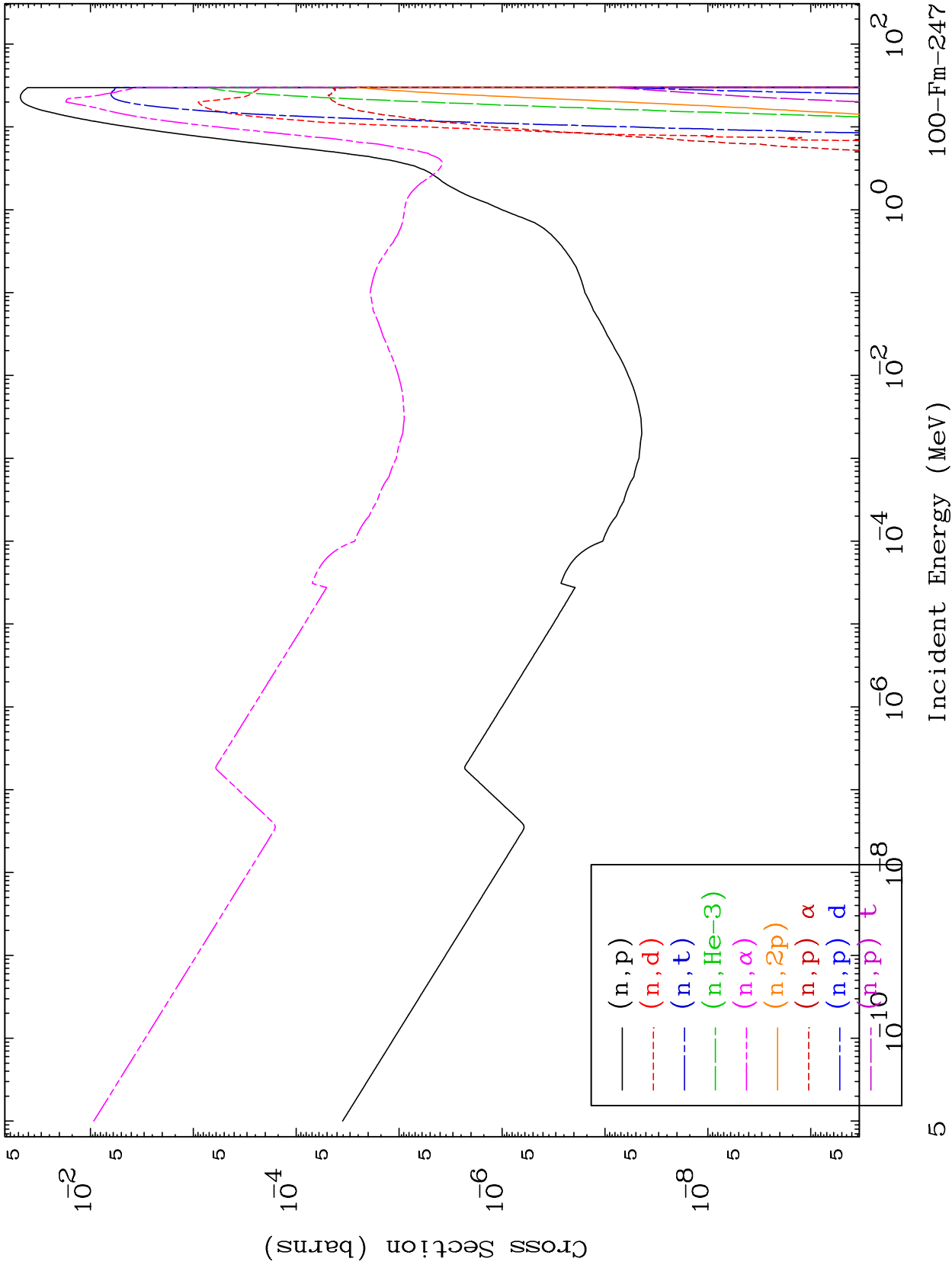
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MAT 9933

Charged Particle
293 Kelvin Cross Sections

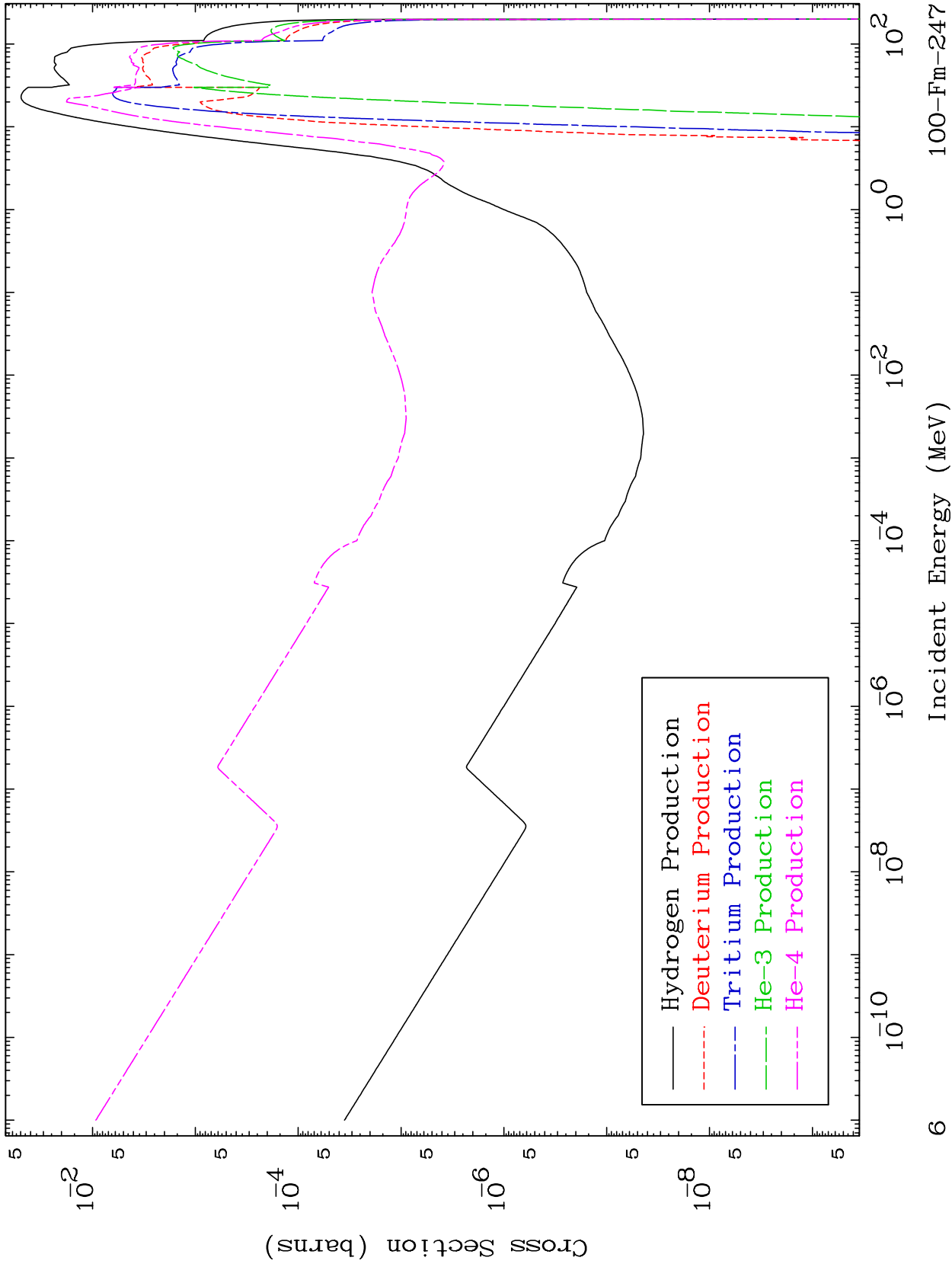
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MAT 9933

Particle Production
293 Kelvin Cross Sections

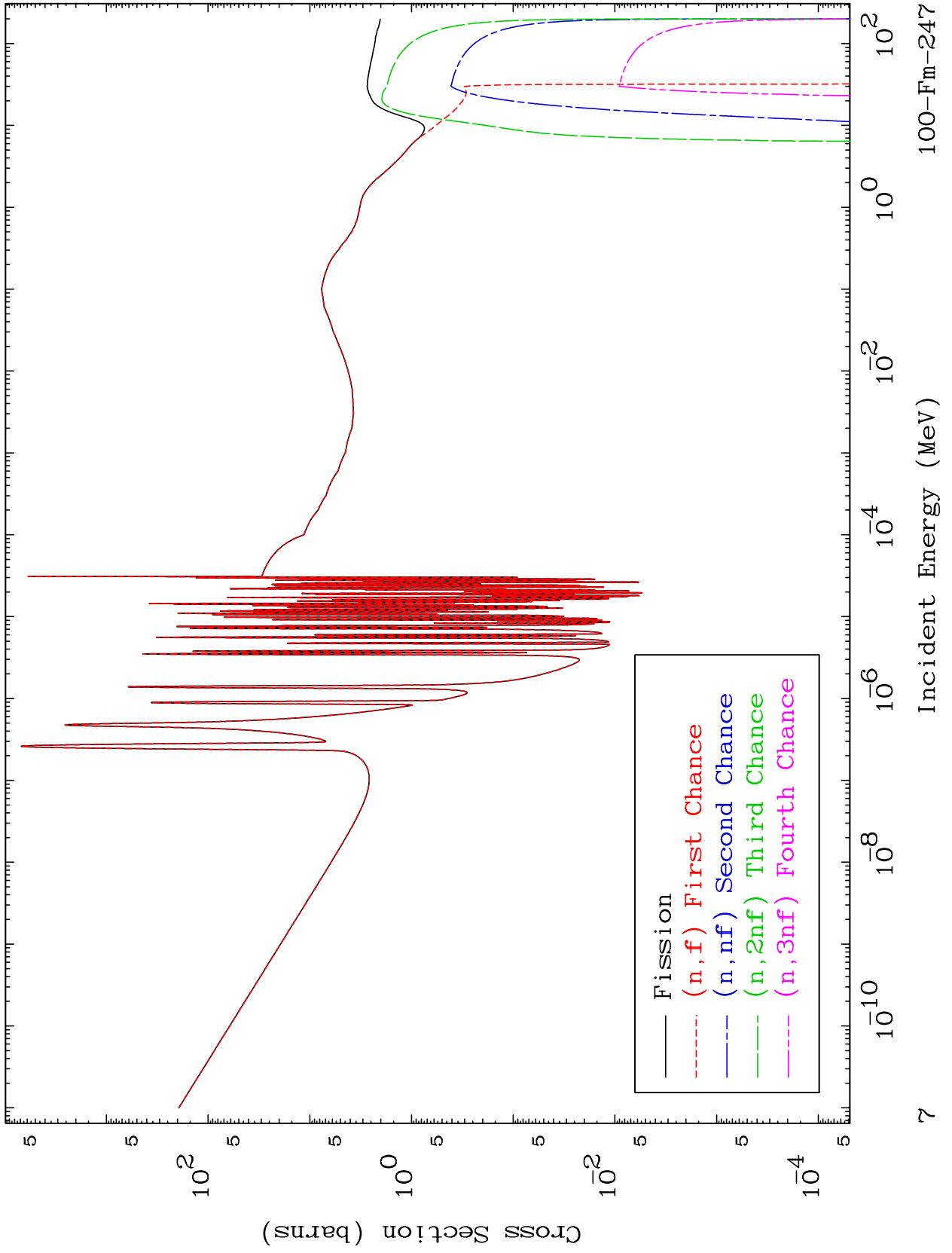
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MAT 9933

Fission
293 Kelvin Cross Sections

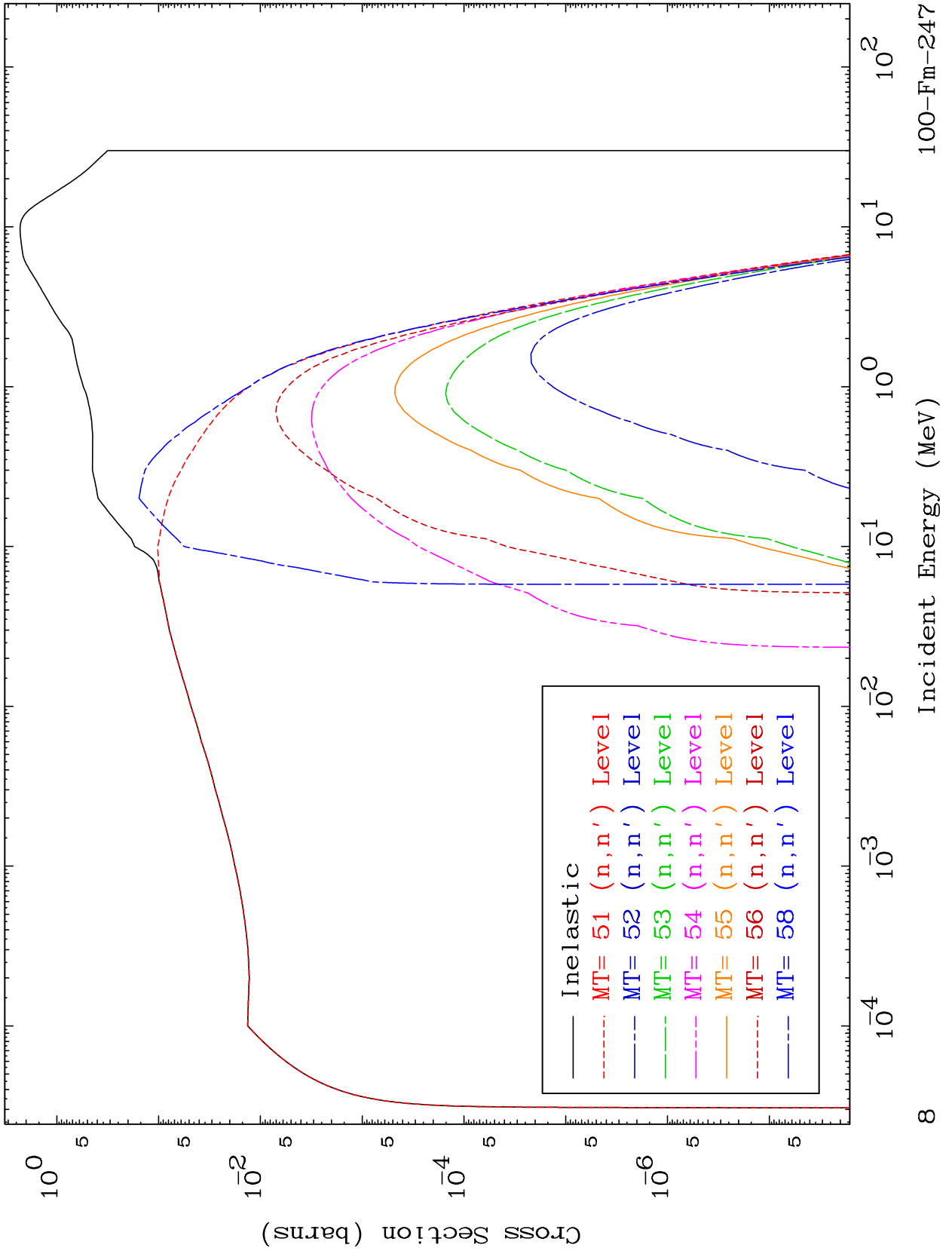
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MAT 9933

(n,n') Level
293 Kelvin Cross Sections

100-Fm-247

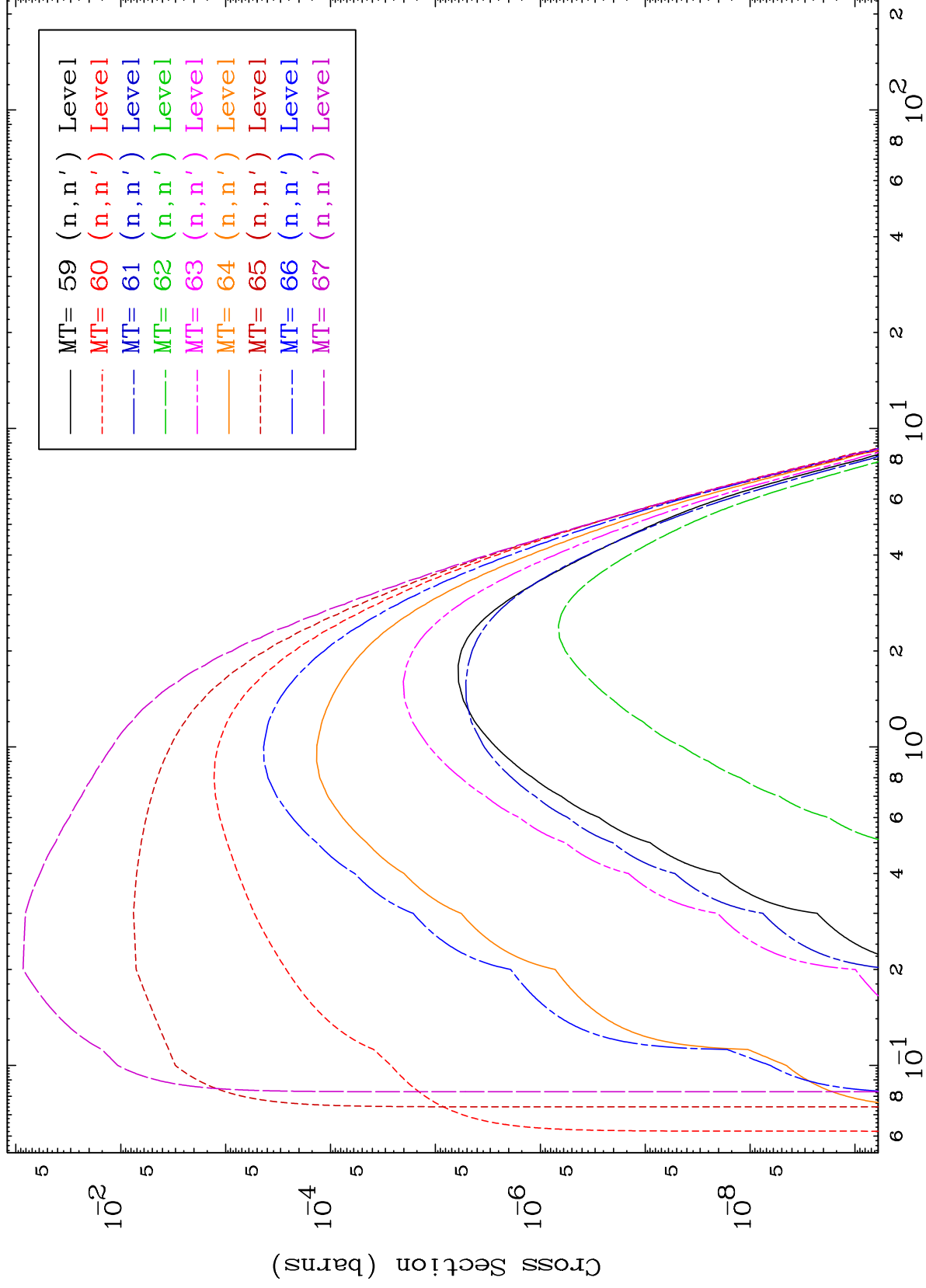


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(n,n') Level

100-Fm-247

293 Kelvin Cross Sections



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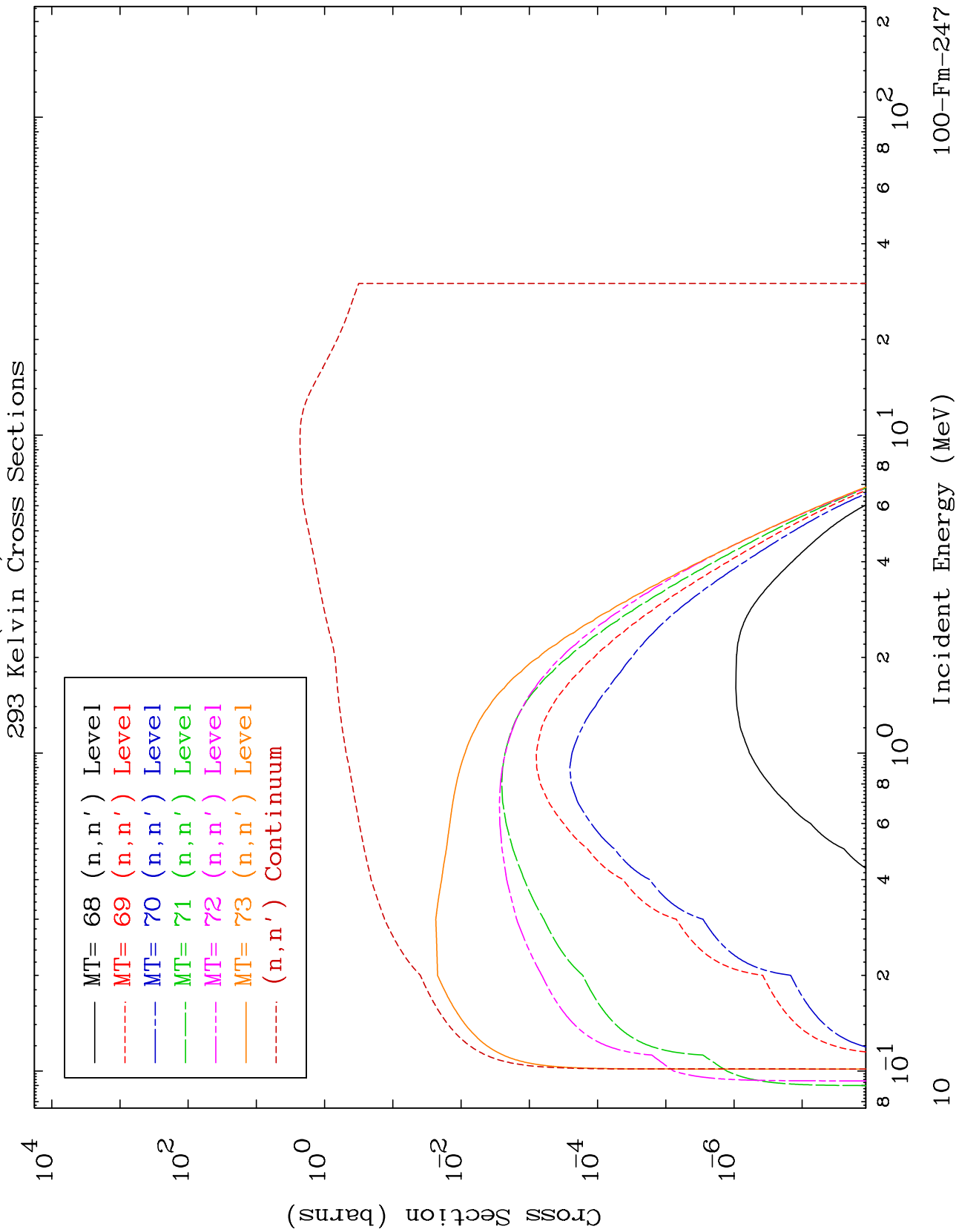
Incident Energy (MeV)

100-Fm-247

MAT 9933

(n,n') Level

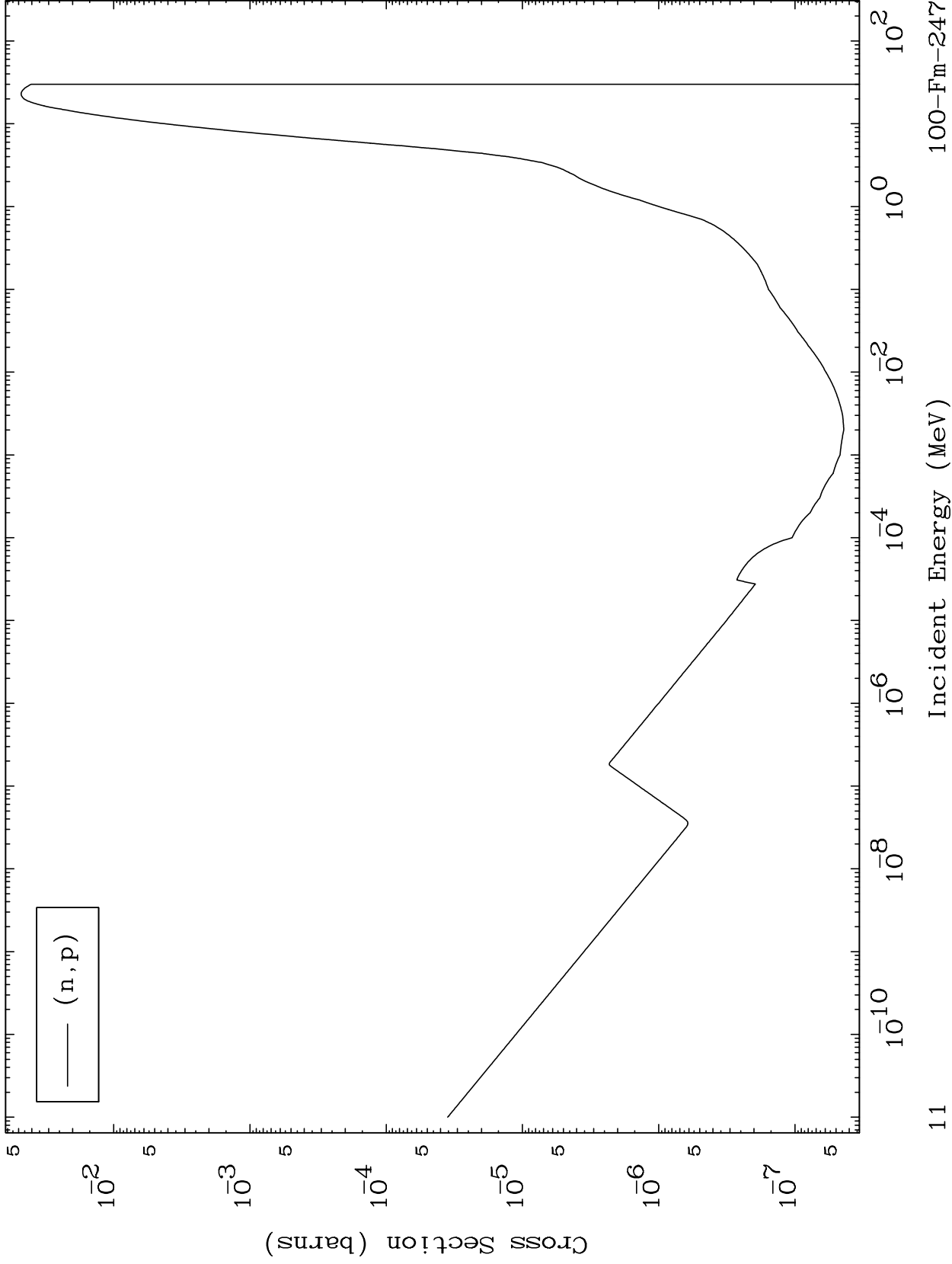
100-Fm-247



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(n,p) Levels
293 Kelvin Cross Sections

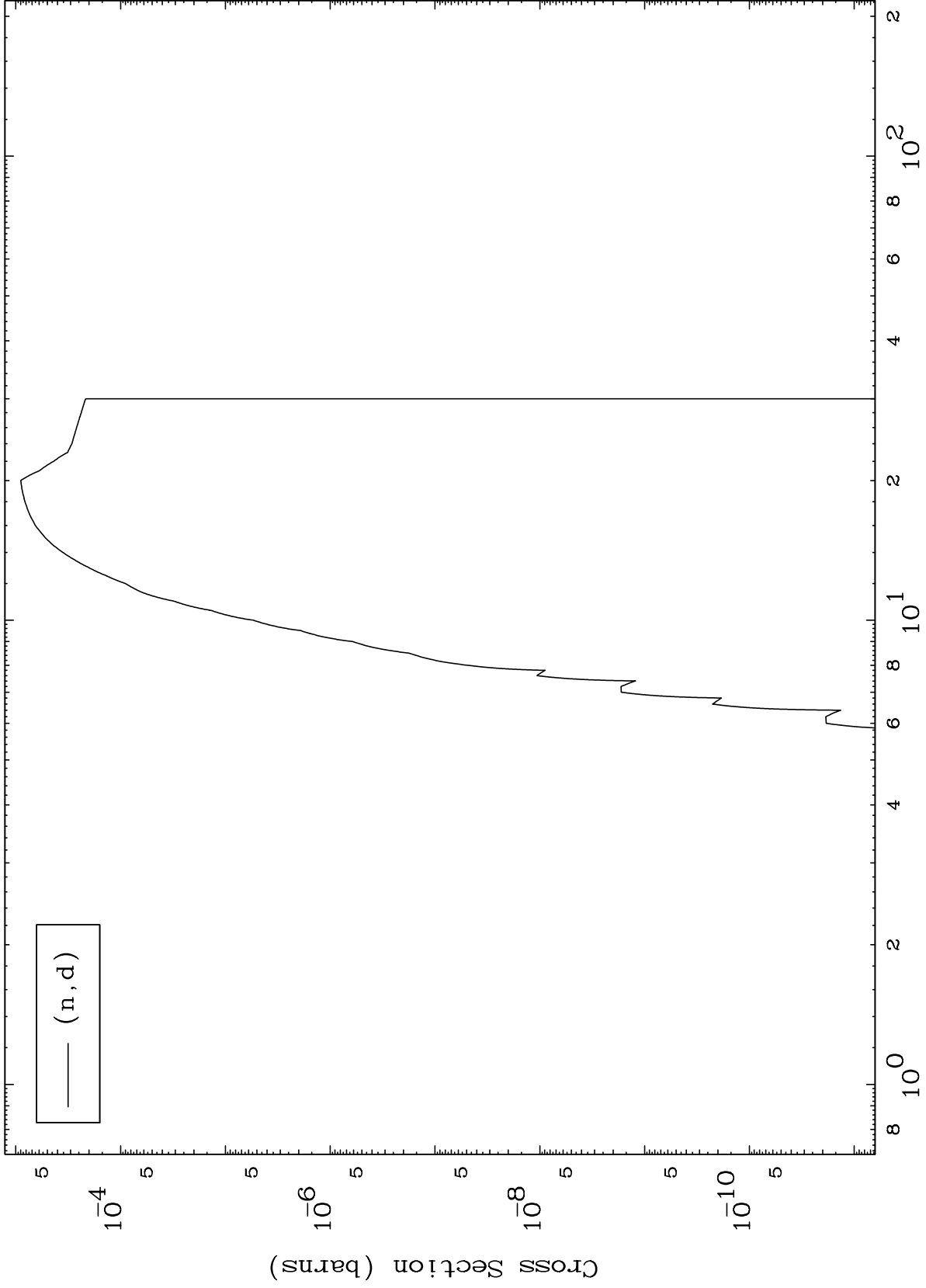
100-Fm-247



MAT 9933

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-247



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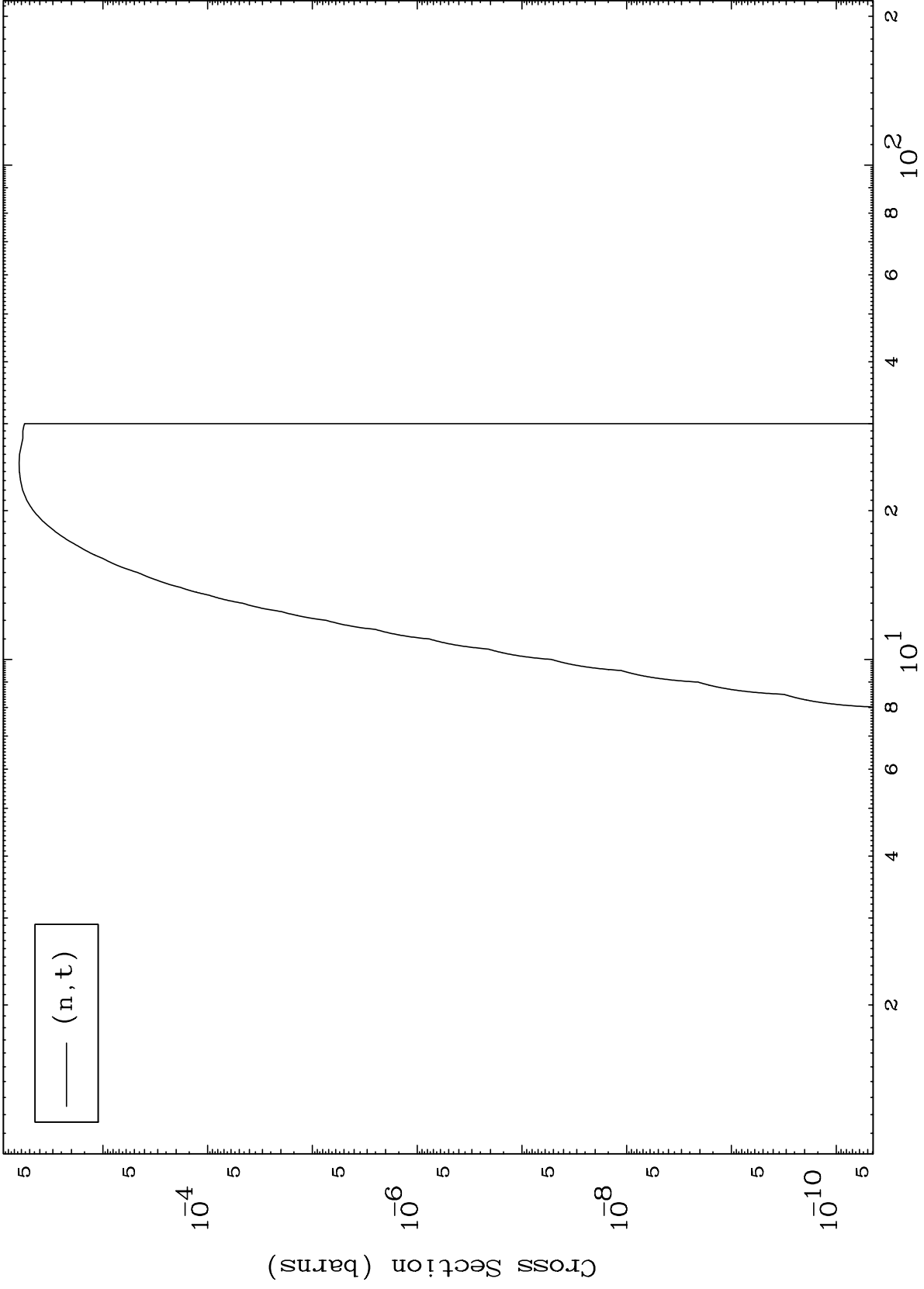
Incident Energy (MeV)

100-Fm-247

MAT 9933

(n,t) Levels
293 Kelvin Cross Sections

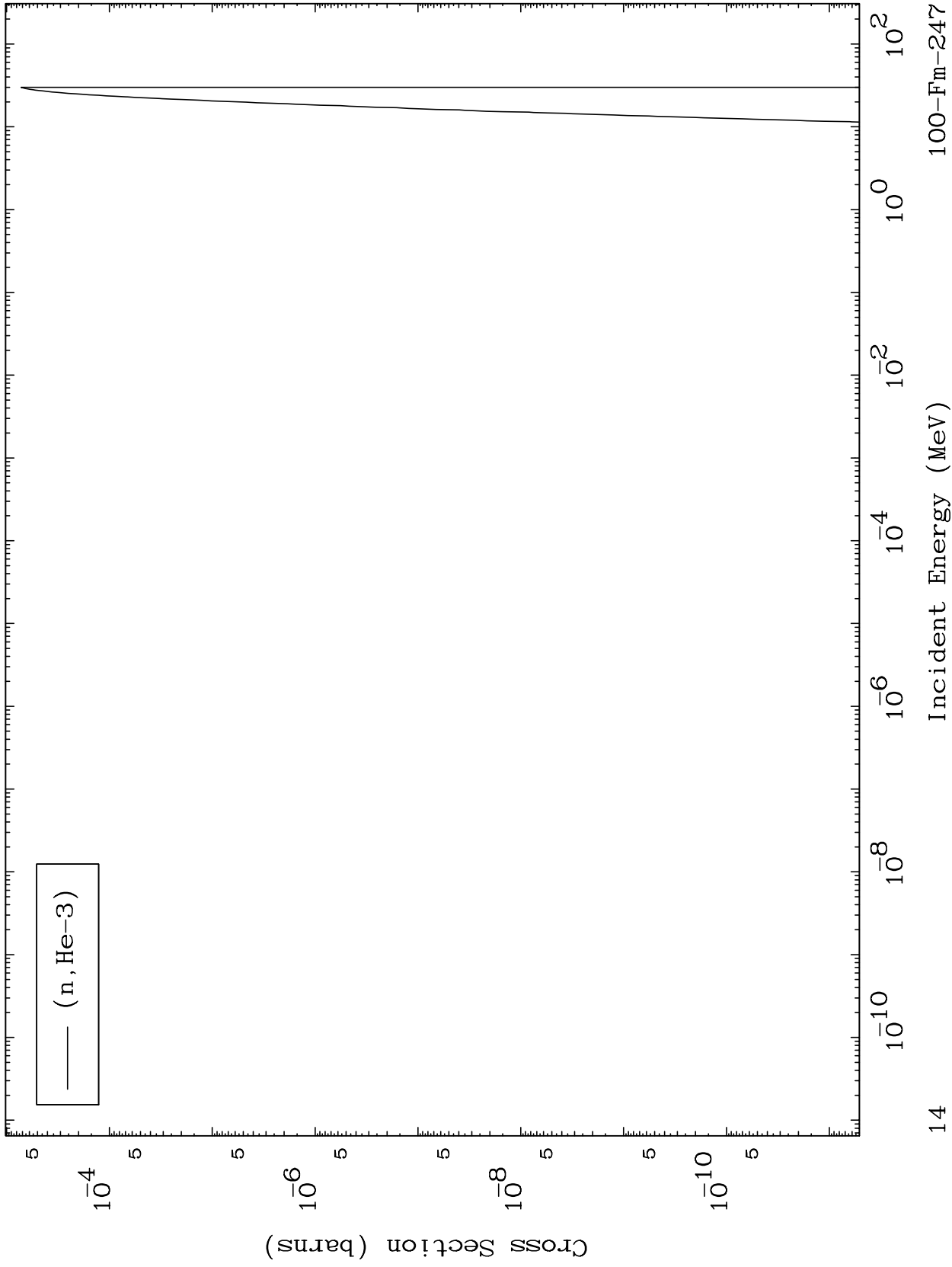
100-Fm-247



MAT 9933

(n,He3) Levels
293 Kelvin Cross Sections

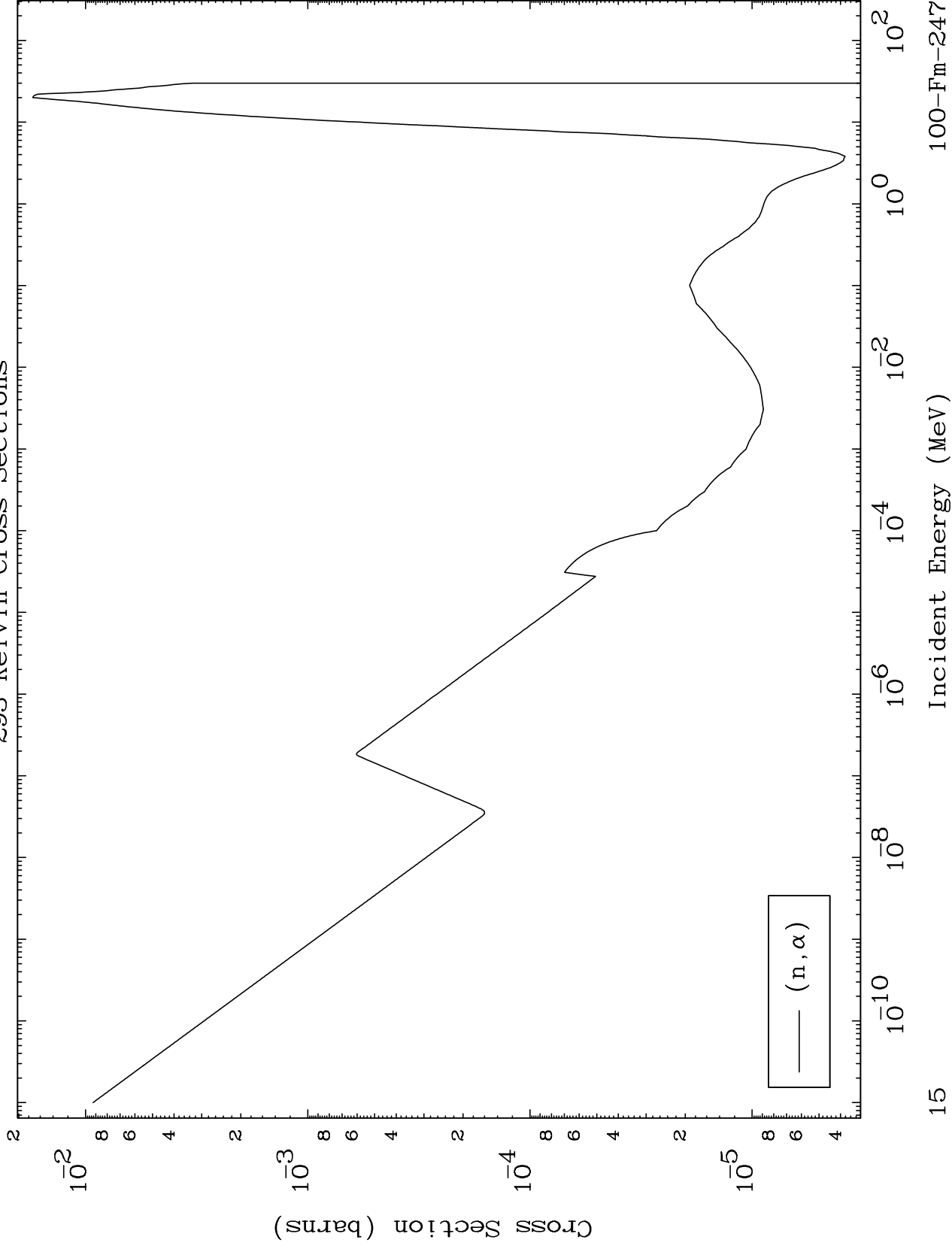
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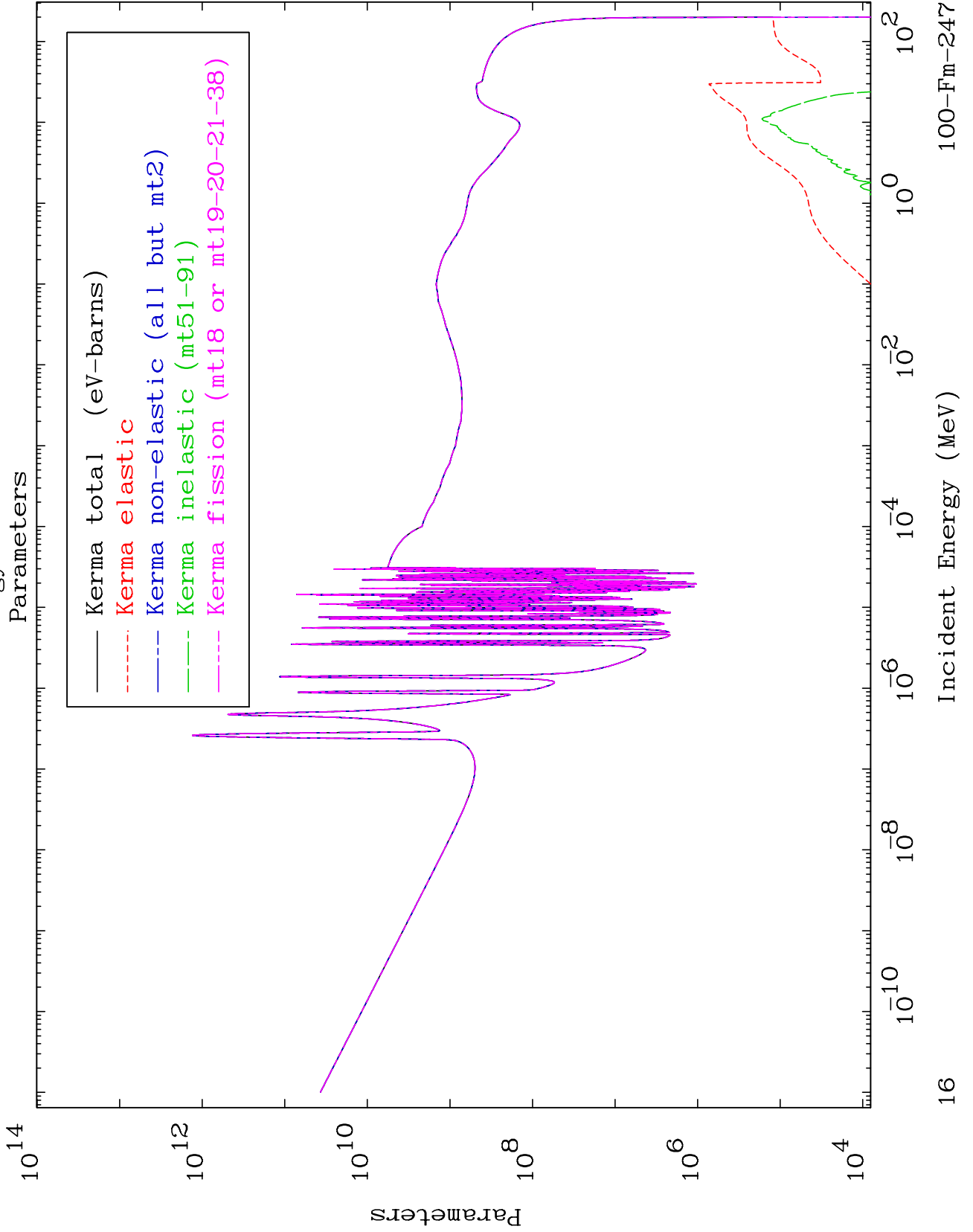


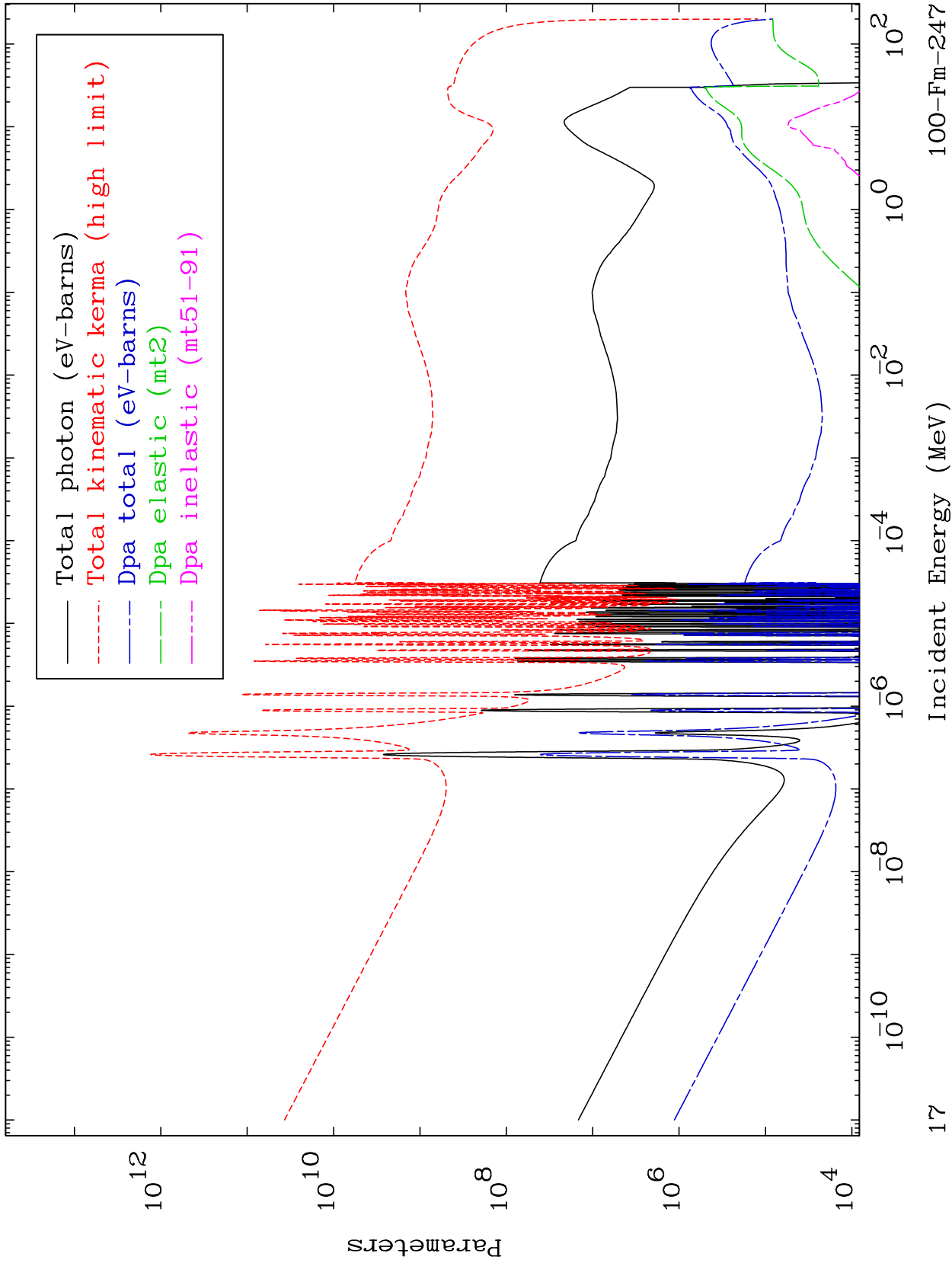
MAT 9933

(n,α) Levels
293 Kelvin Cross Sections

100-Fm-247



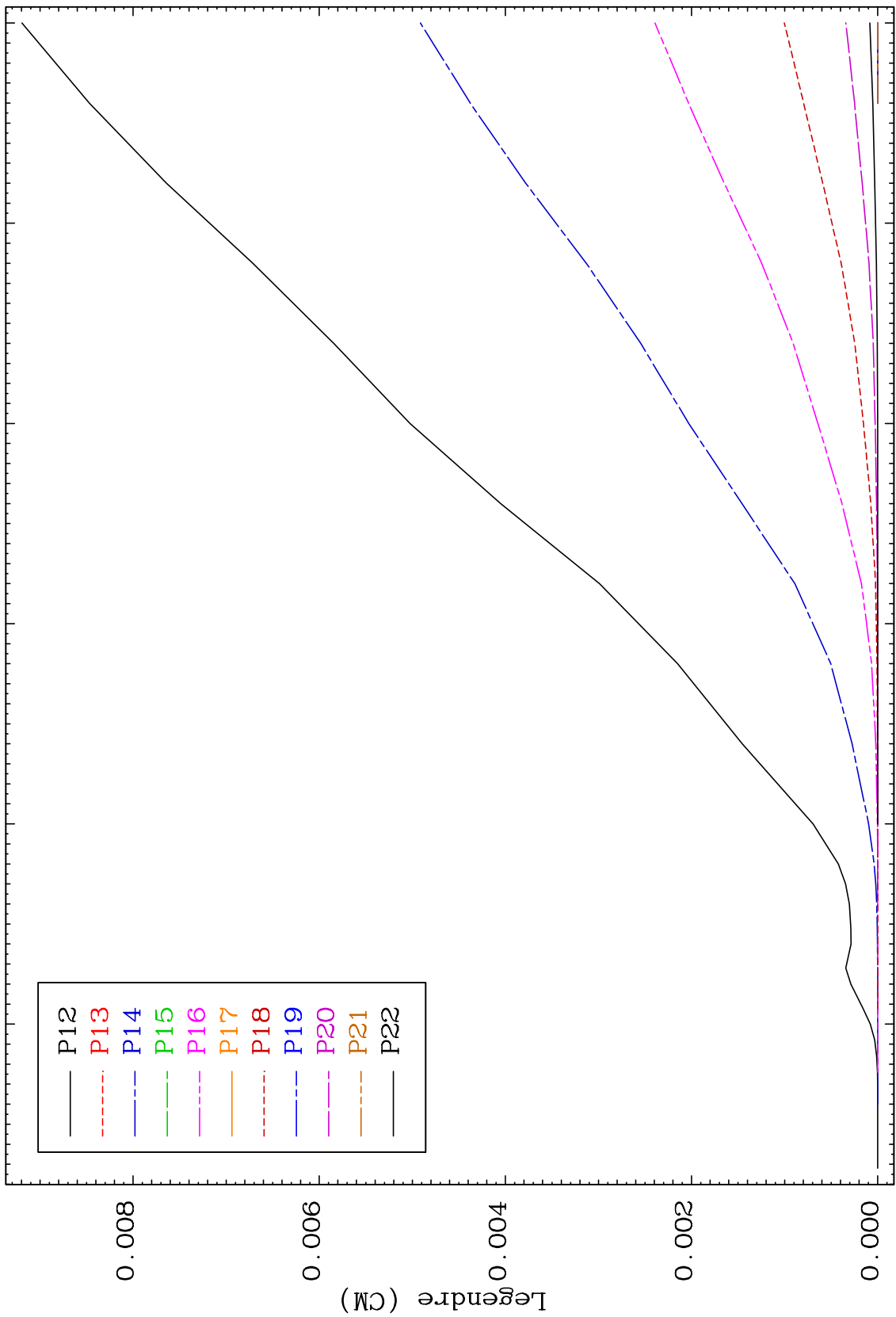




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Elastic Legendre Coefficients

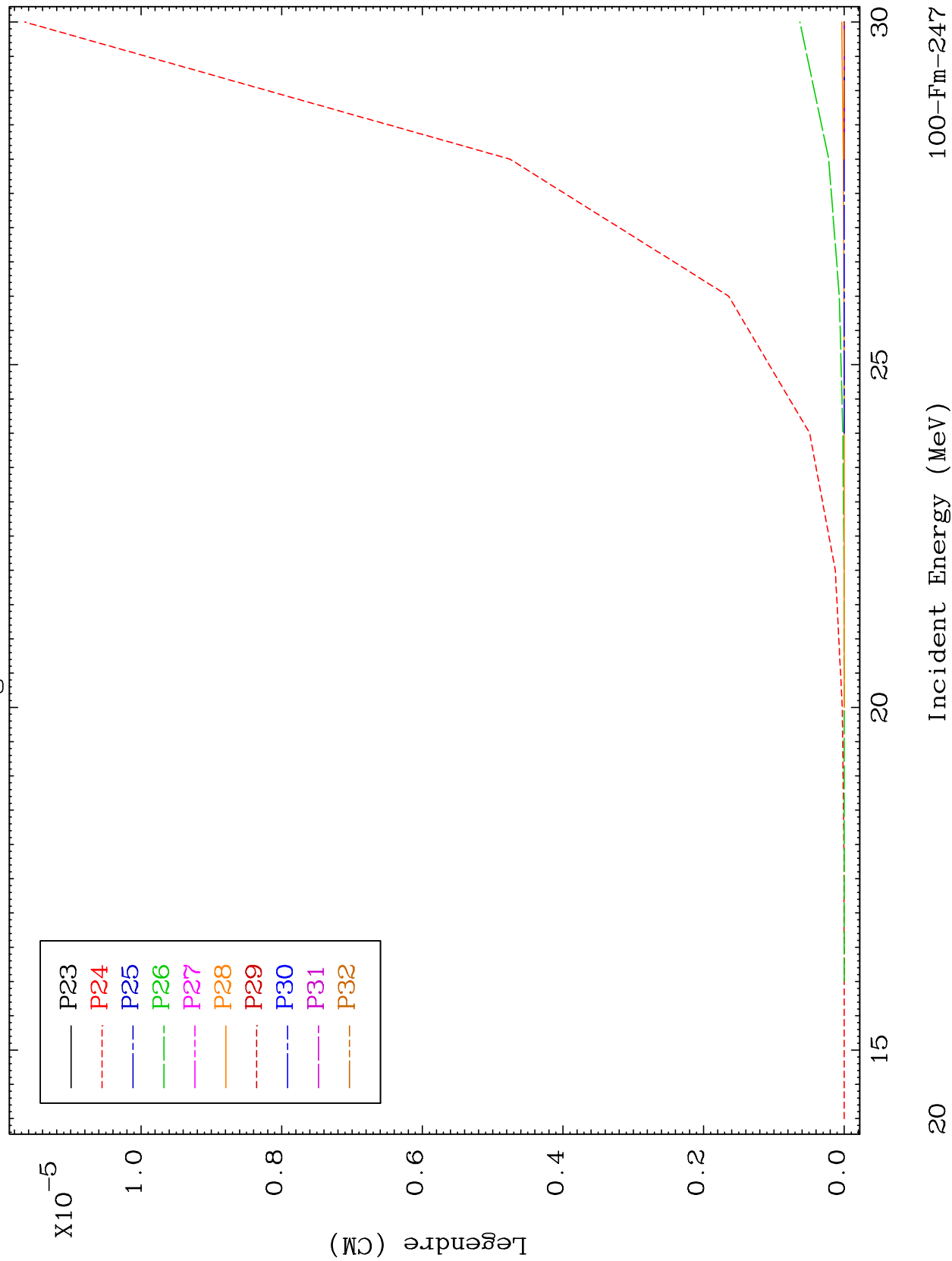
100-Fm-247



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Elastic Legendre Coefficients

100-Fm-247



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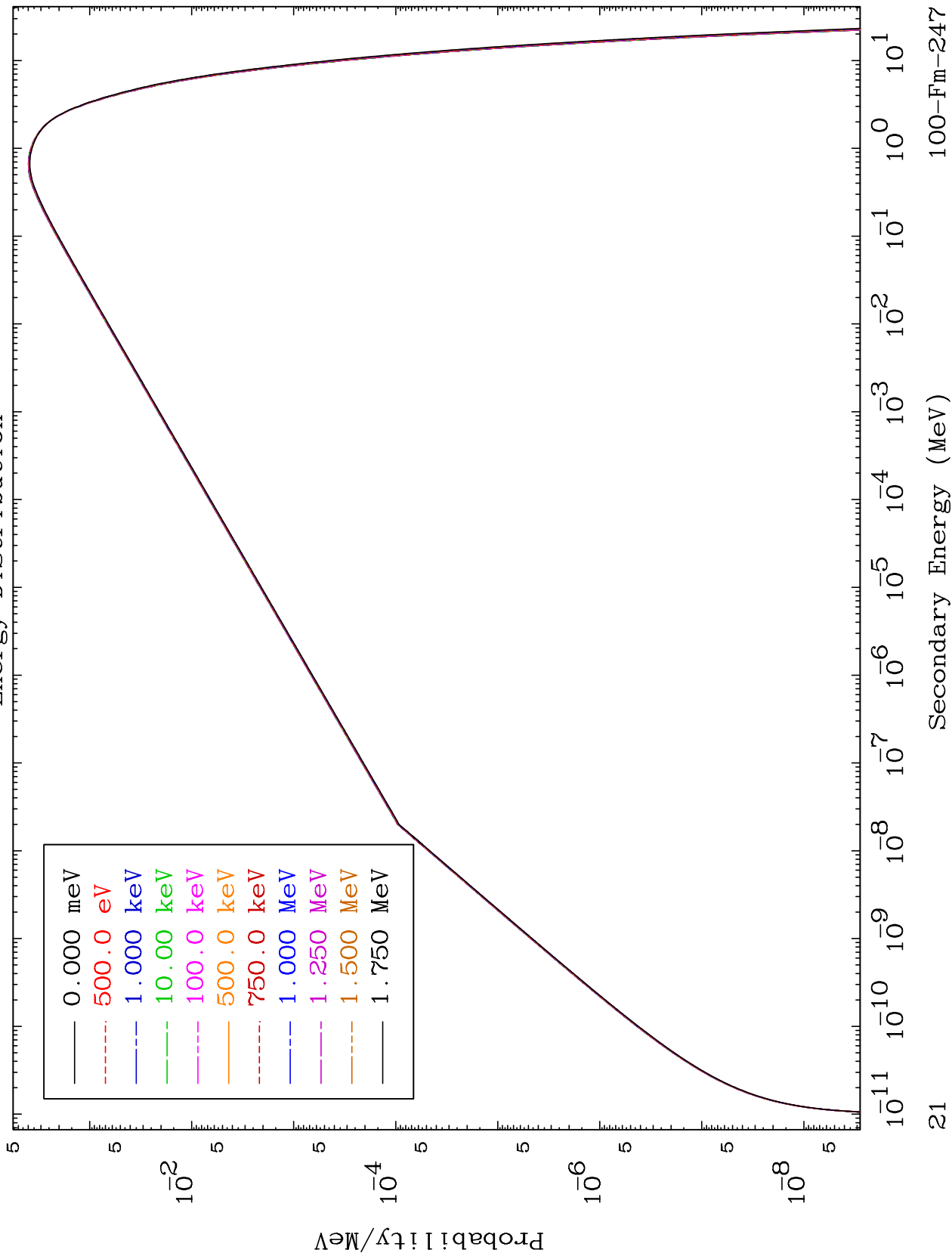
Incident Energy (MeV)

100-Fm-247

MAT 9933

Fission Energy Distribution

100-Fm-247

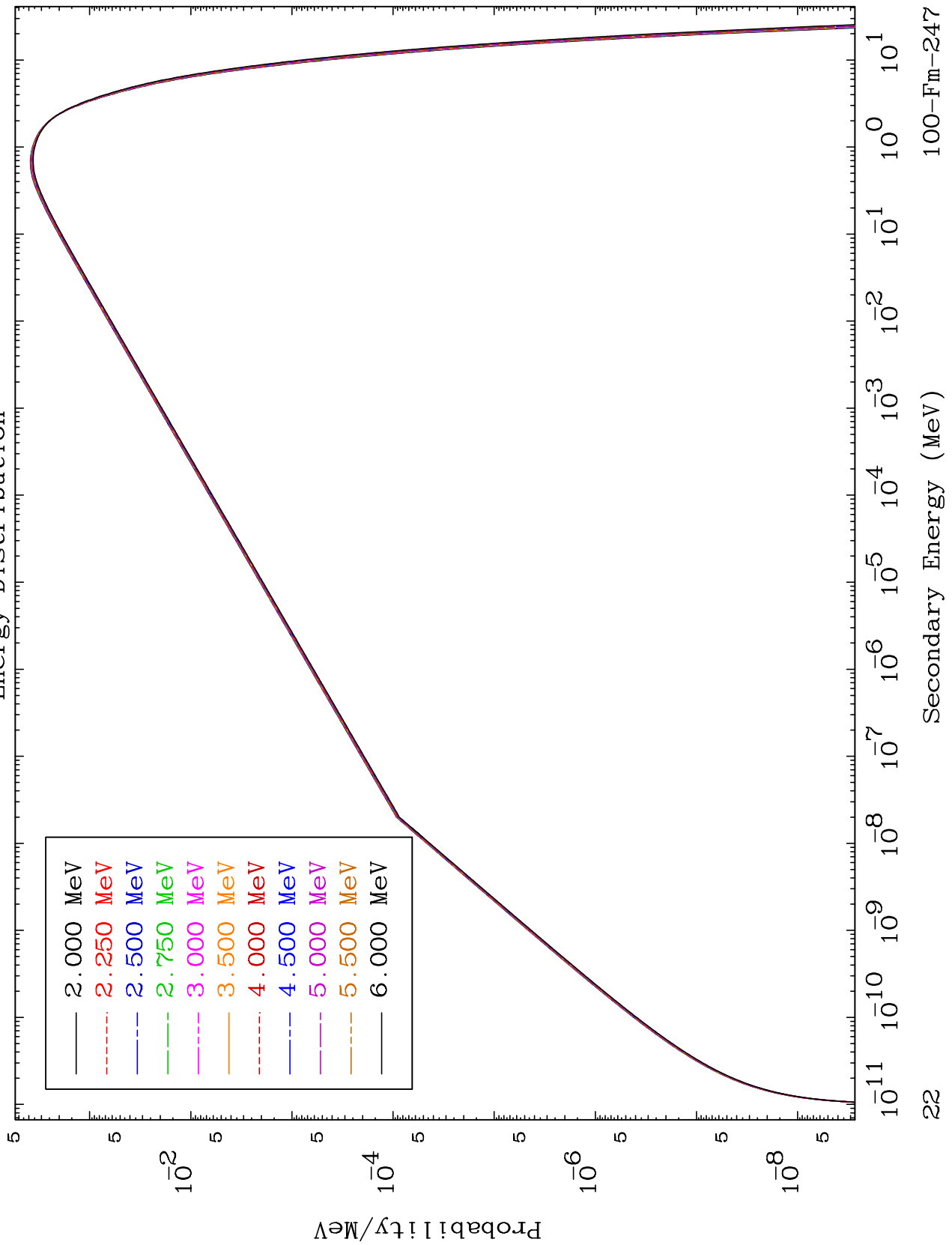


100-Fm-247

MAT 9933

Fission Energy Distribution

100-Fm-247



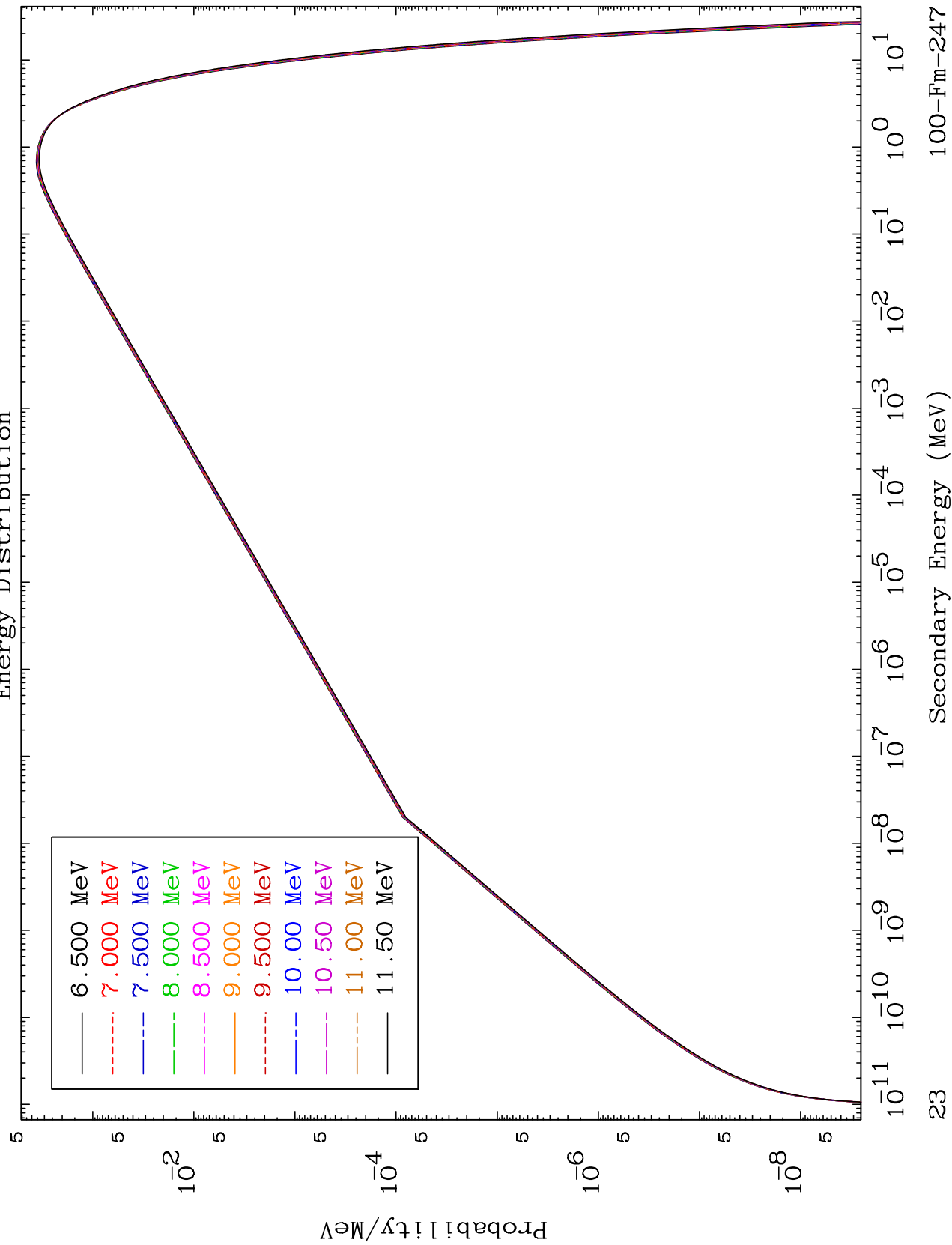
100-Fm-247

Secondary Energy (MeV)

MAT 9933

Fission Energy Distribution

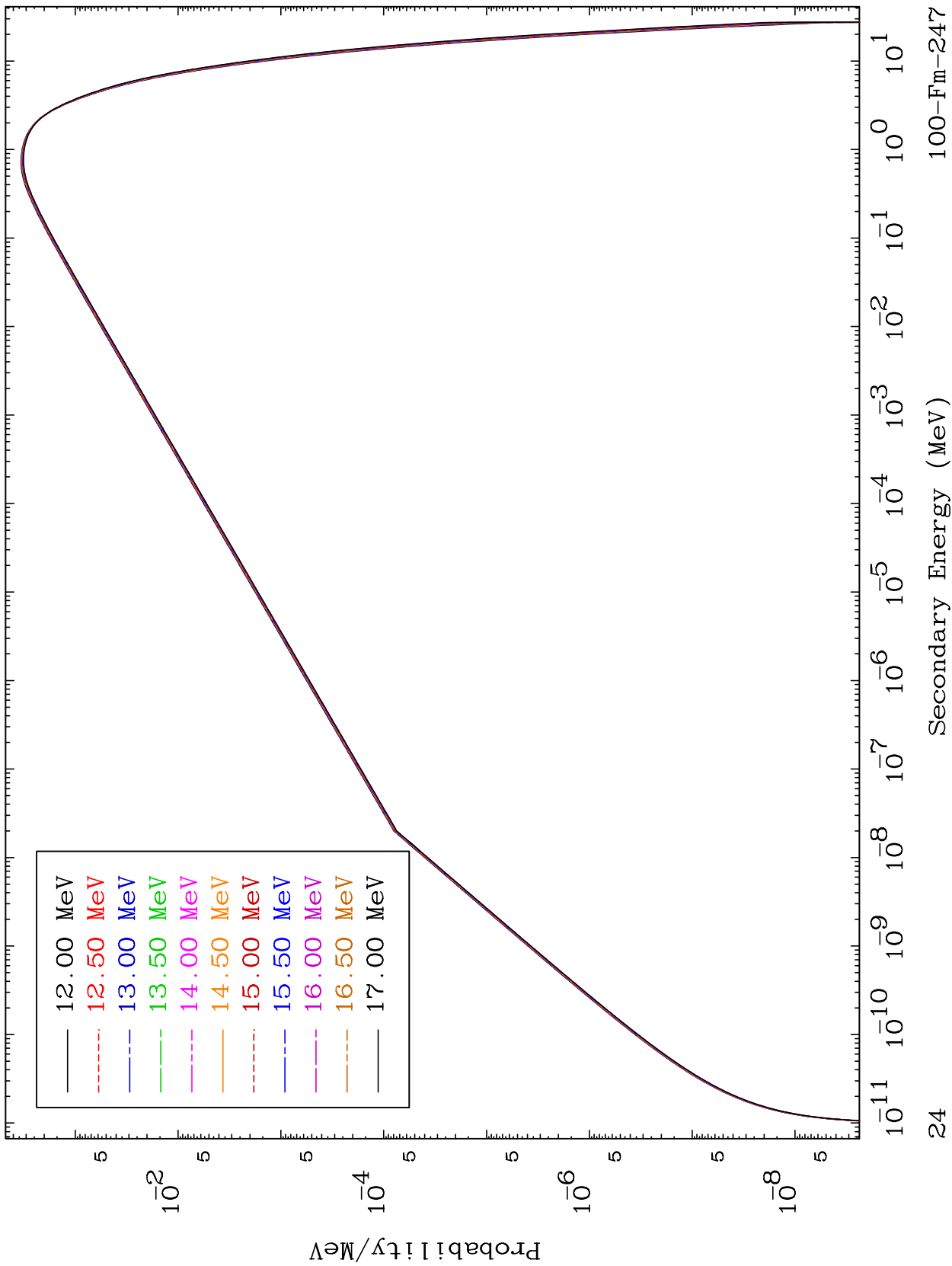
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MAT 9933

Fission Energy Distribution

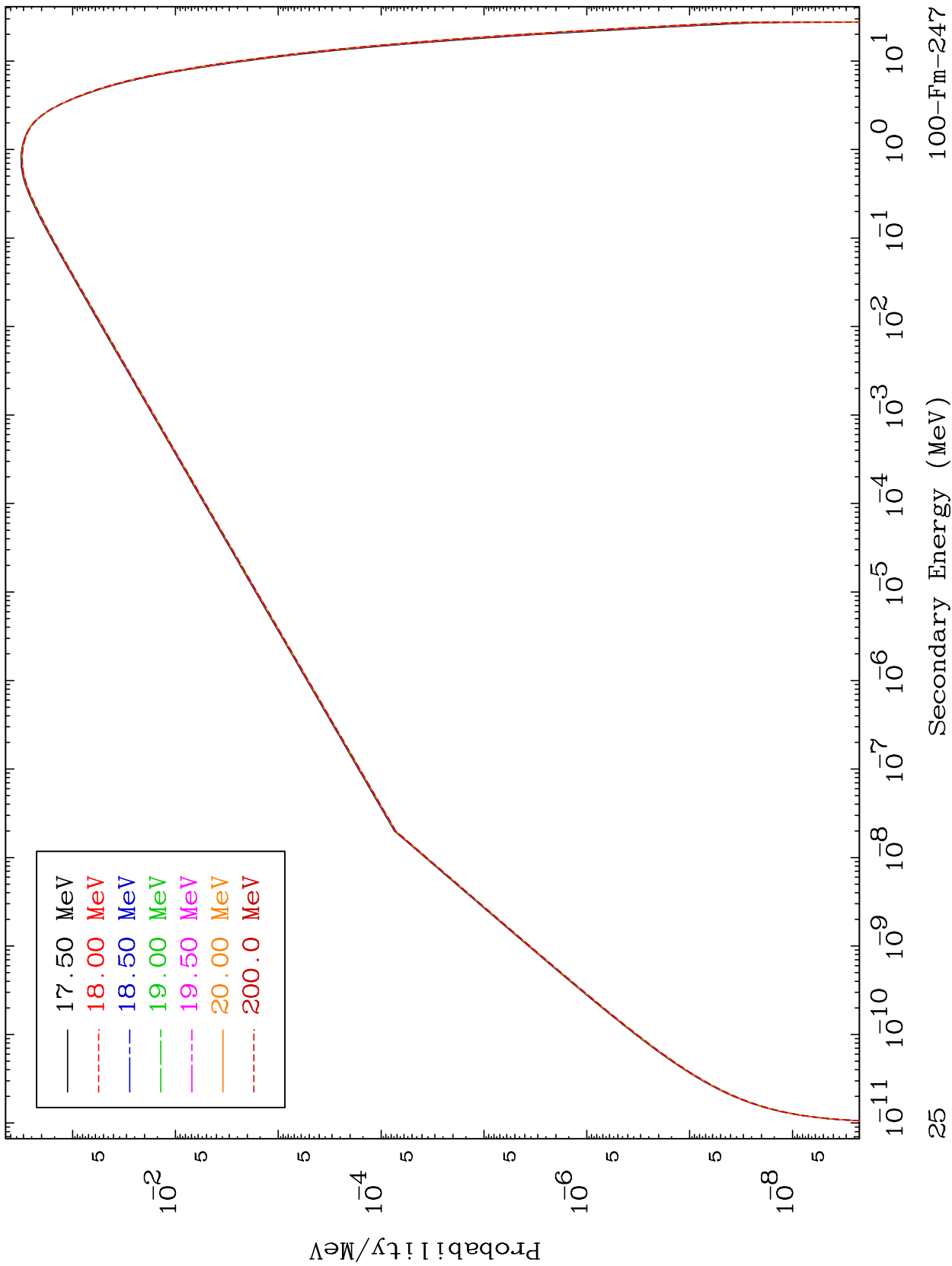
100-Fm-247



MAT 9933

Fission Energy Distribution

100-Fm-247



Inelastic
Radionuclide Production Cross Section

