

Program EVALPLOT  
(Version 2018-1)

by

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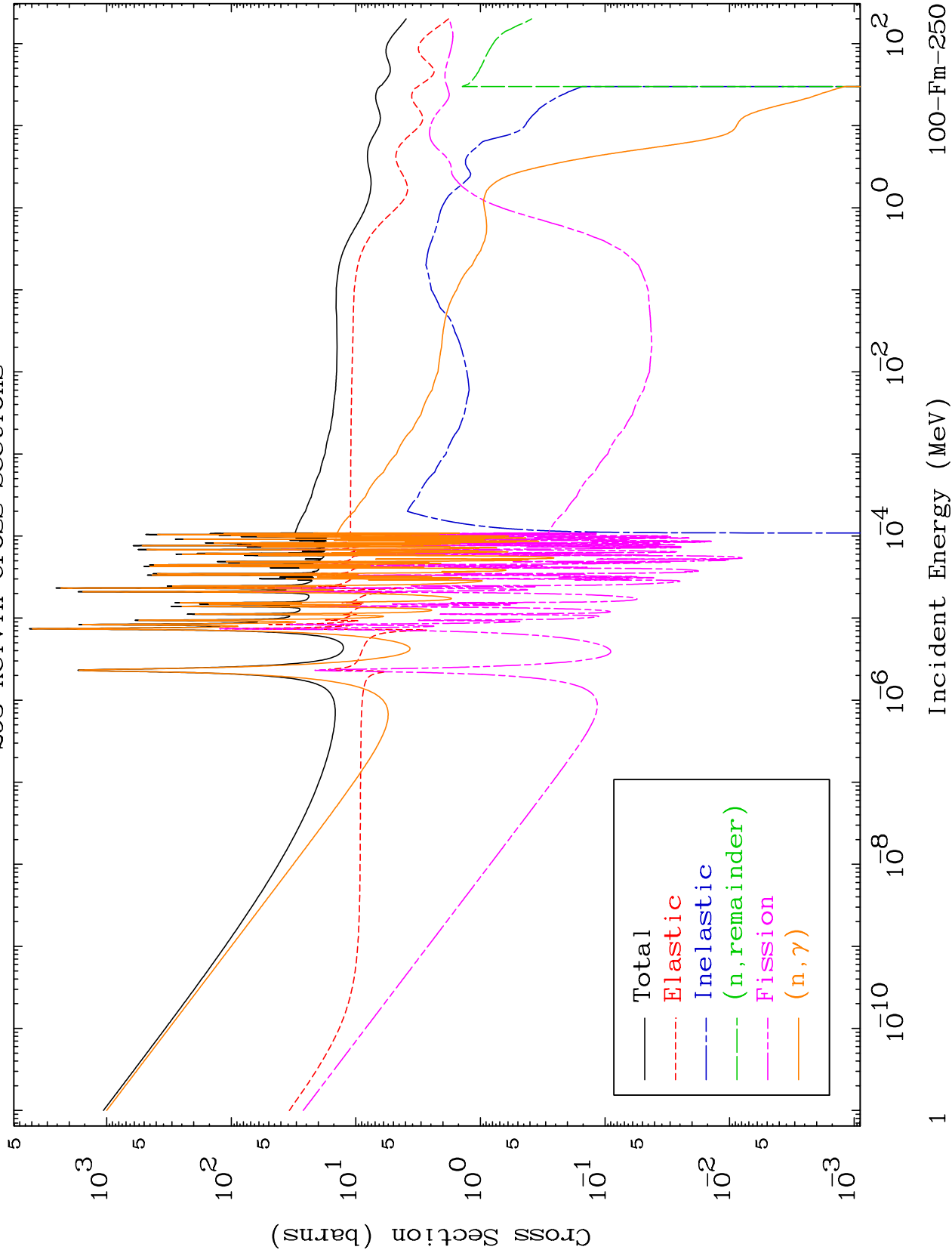
Press Mouse Button to Start

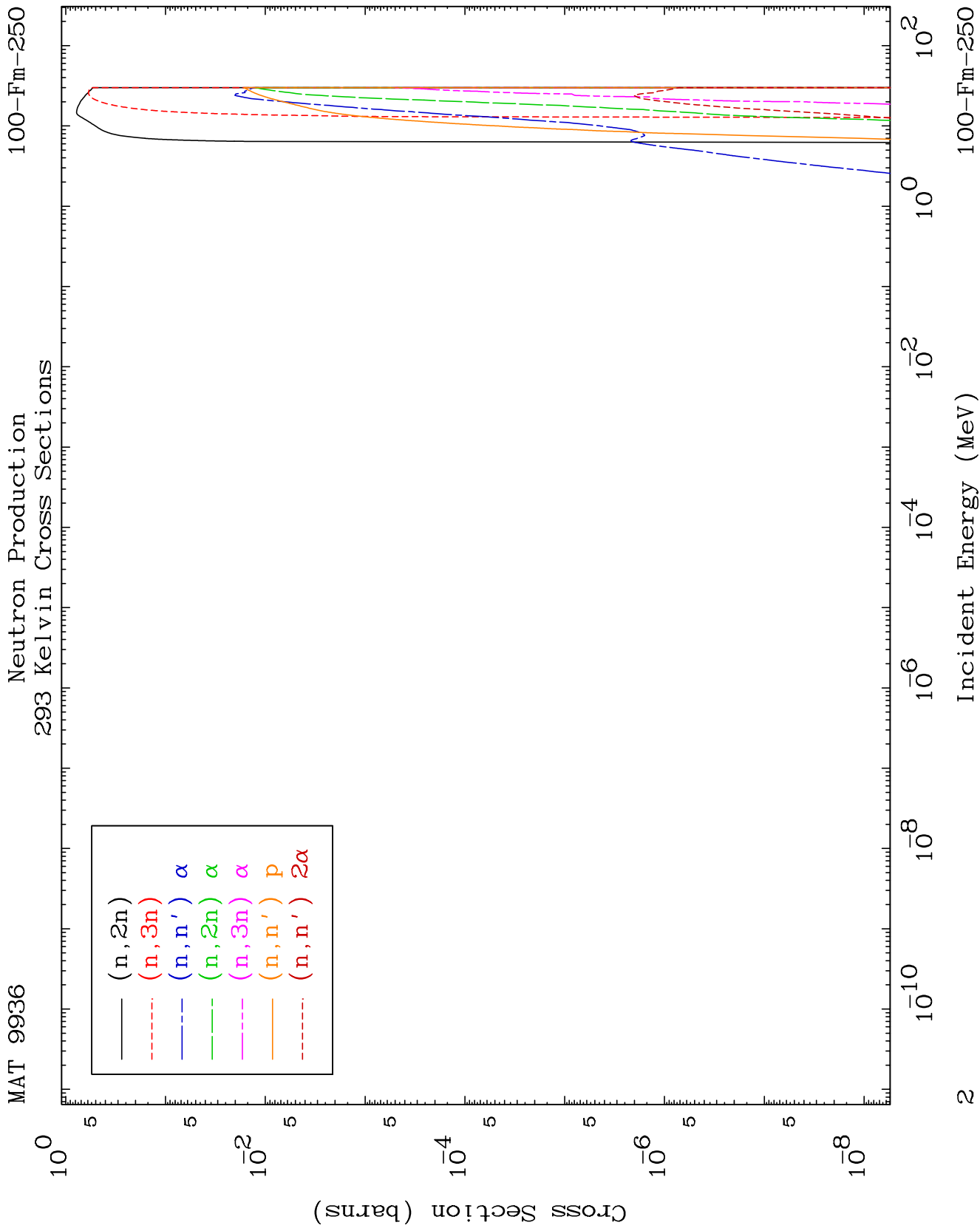
MAT 9936

Major

293 Kelvin Cross Sections

100-Fm-250

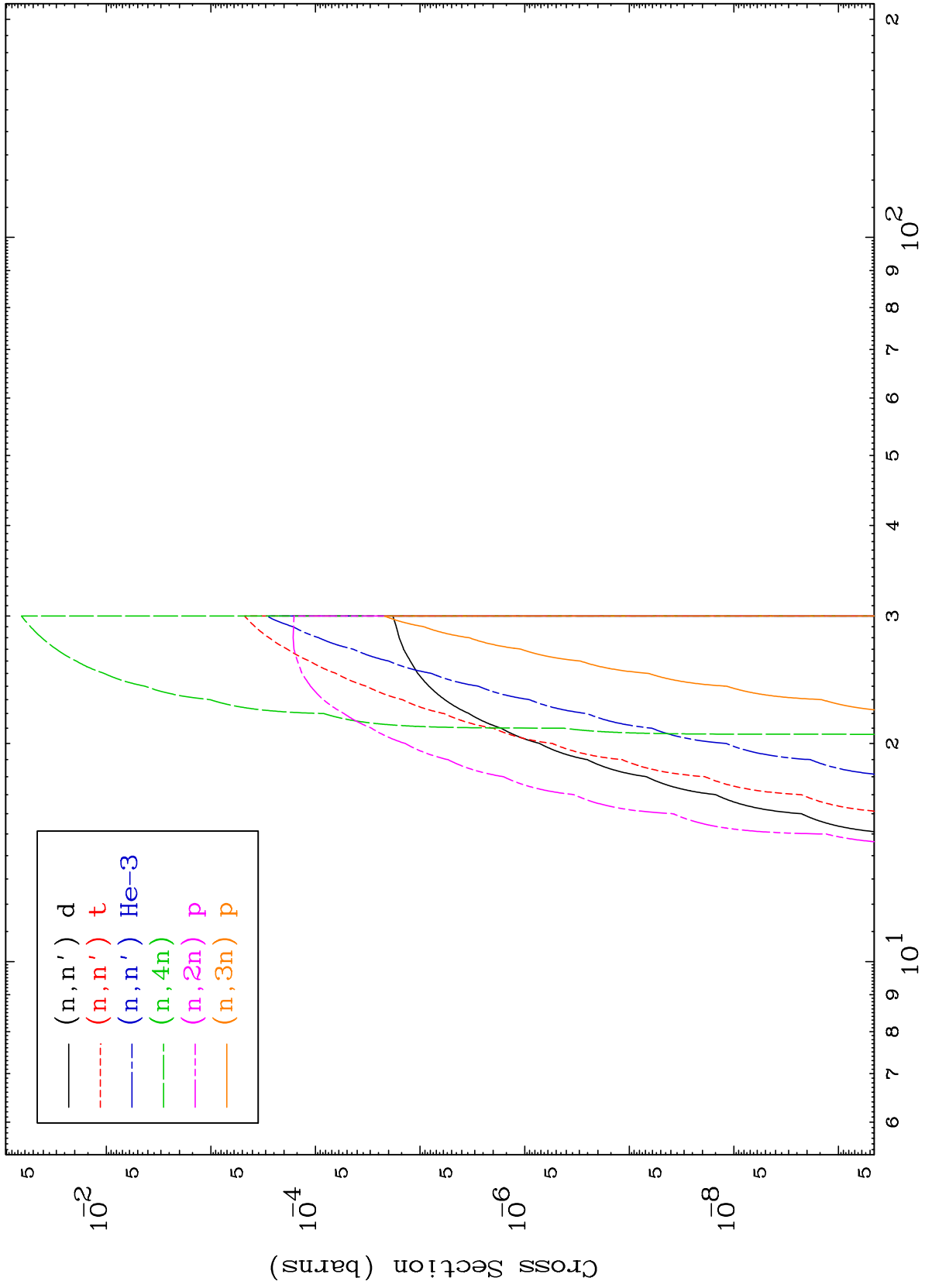




MAT 9936

Neutron Production  
293 Kelvin Cross Sections

100-Fm-250



3

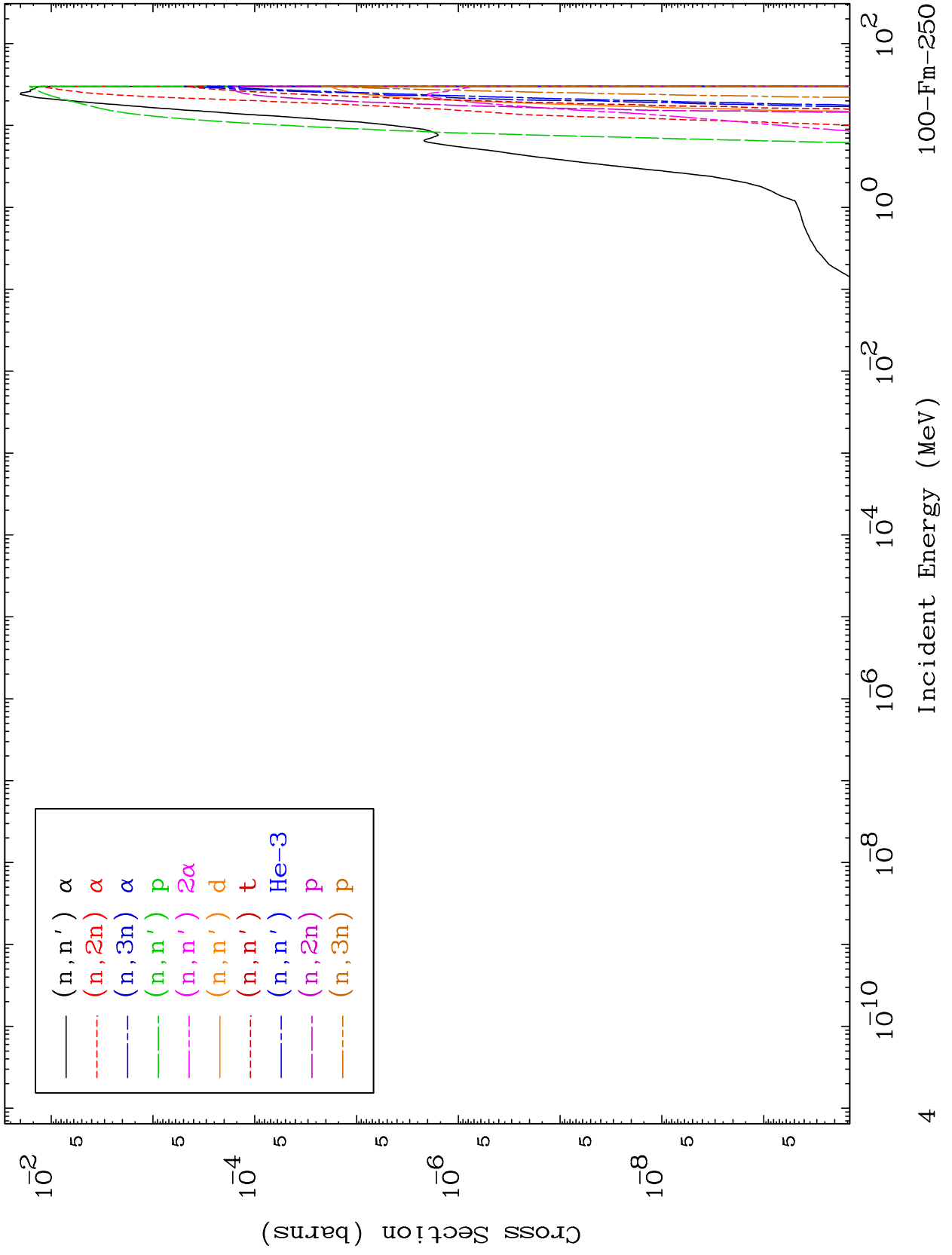
Incident Energy (MeV)

100-Fm-250

MAT 9936

Charged Particle  
293 Kelvin Cross Sections

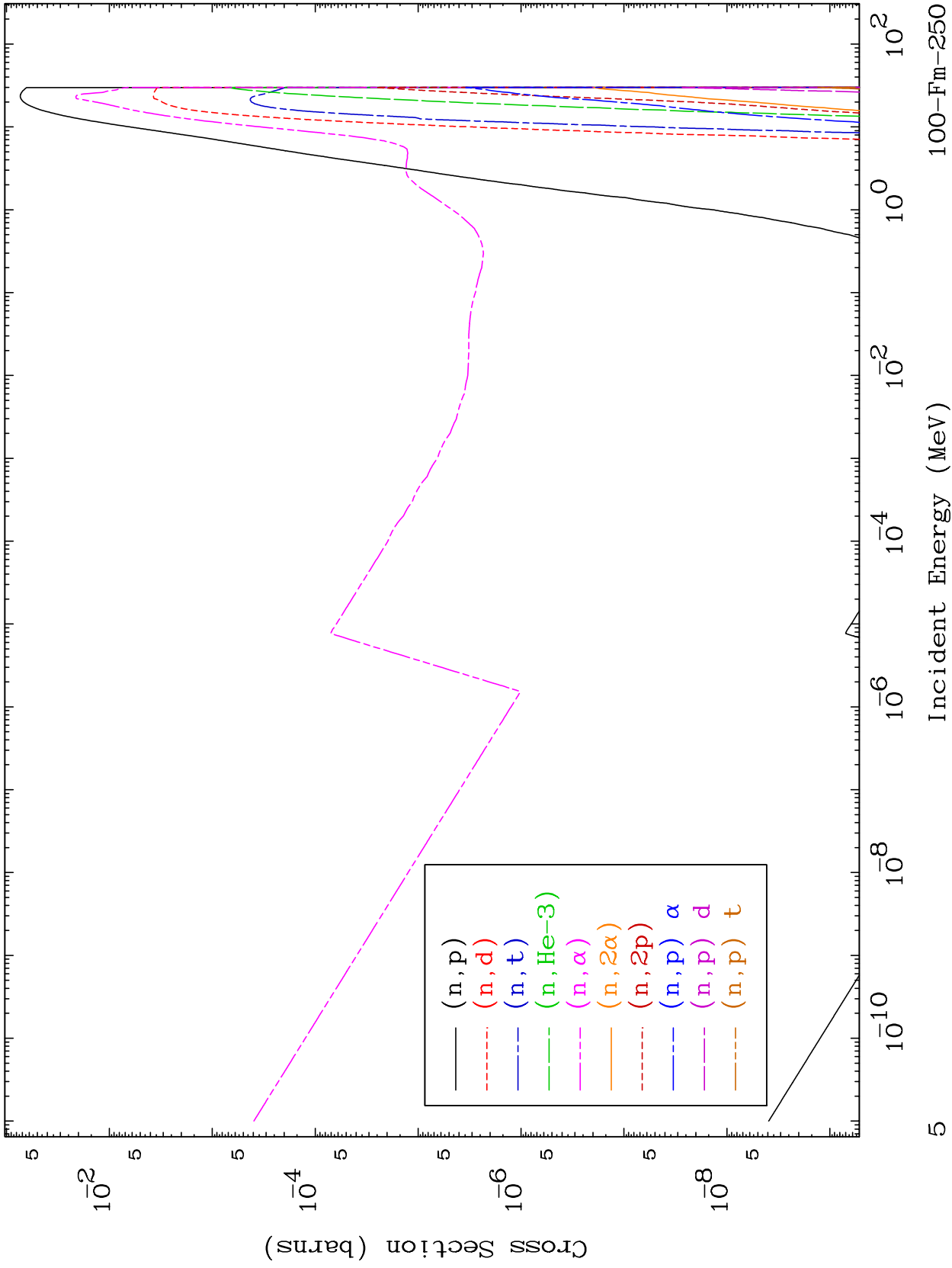
100-Fm-250



MAT 9936

Charged Particle  
293 Kelvin Cross Sections

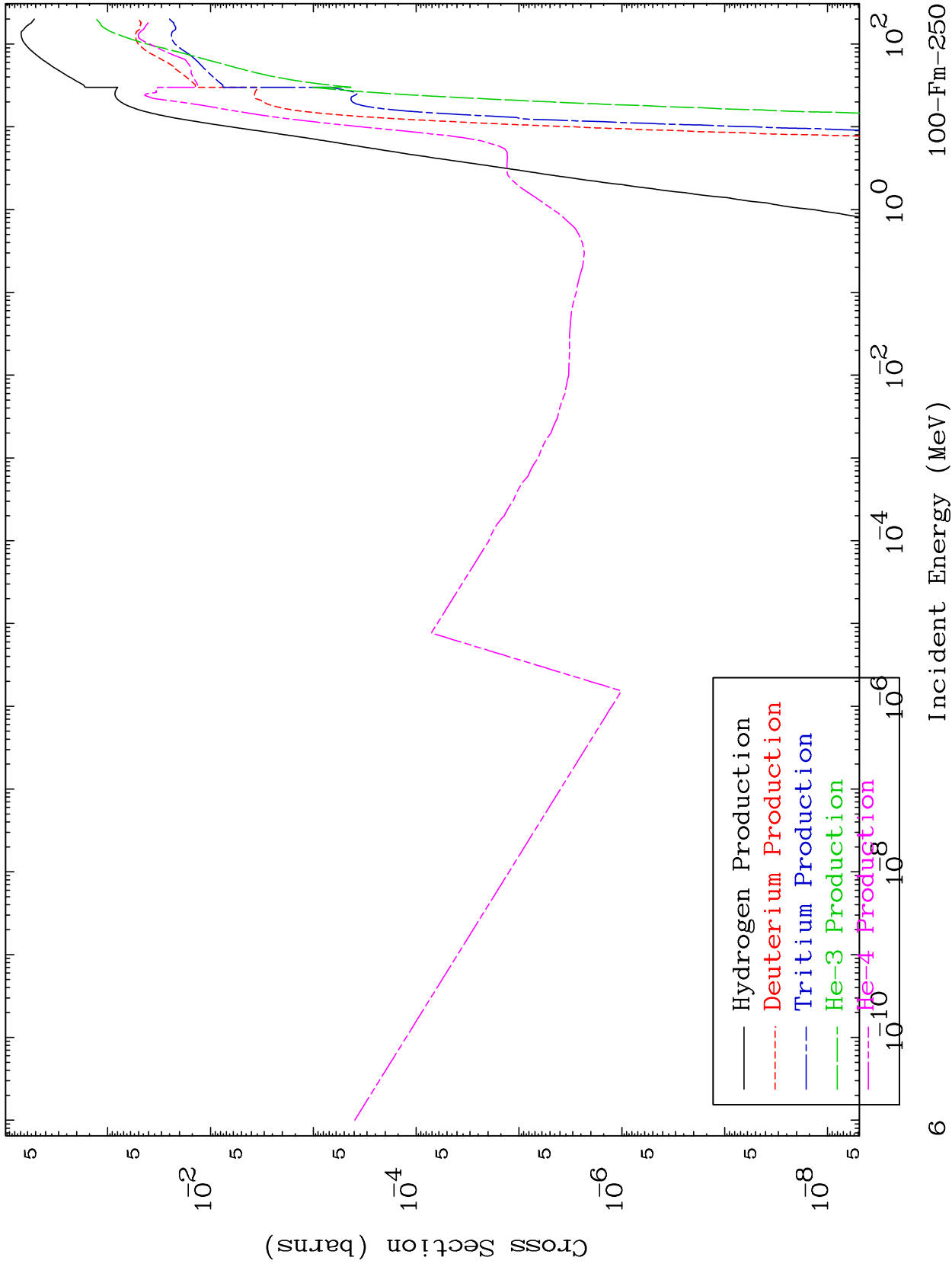
100-Fm-250



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Particle Production  
293 Kelvin Cross Sections

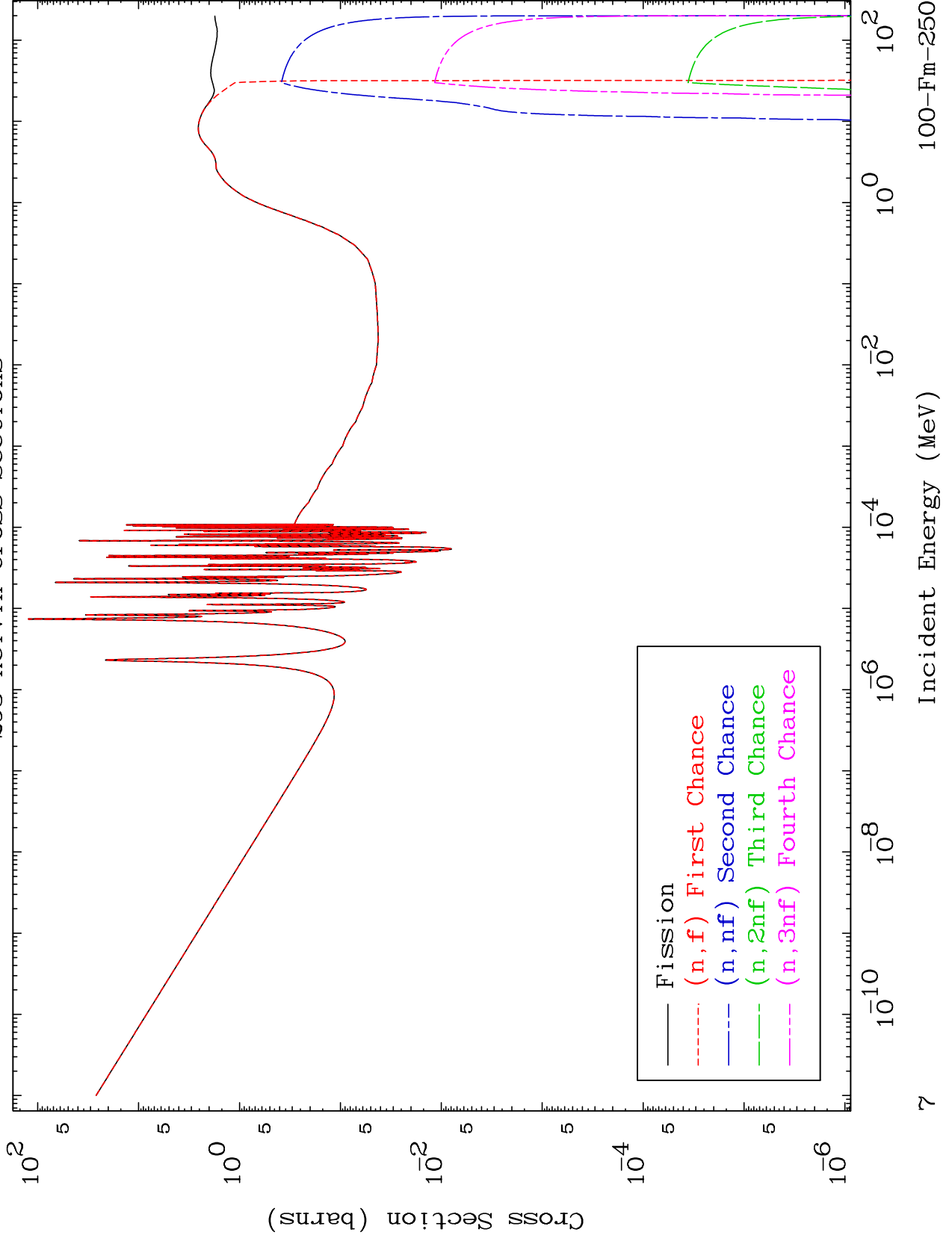
100-Fm-250



MAT 9936

Fission  
293 Kelvin Cross Sections

100-Fm-250



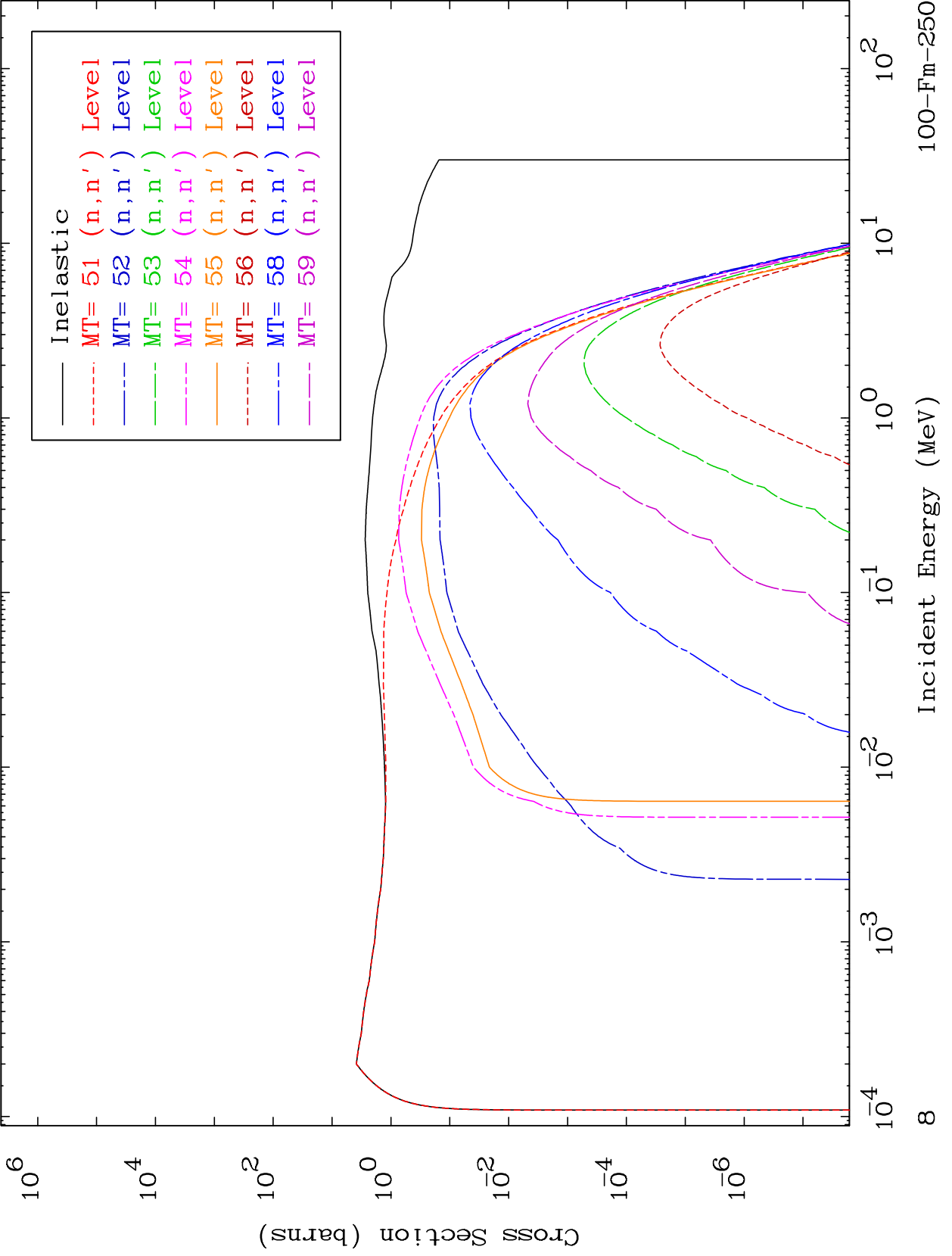


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(n,n') Level

293 Kelvin Cross Sections

100-Fm-250

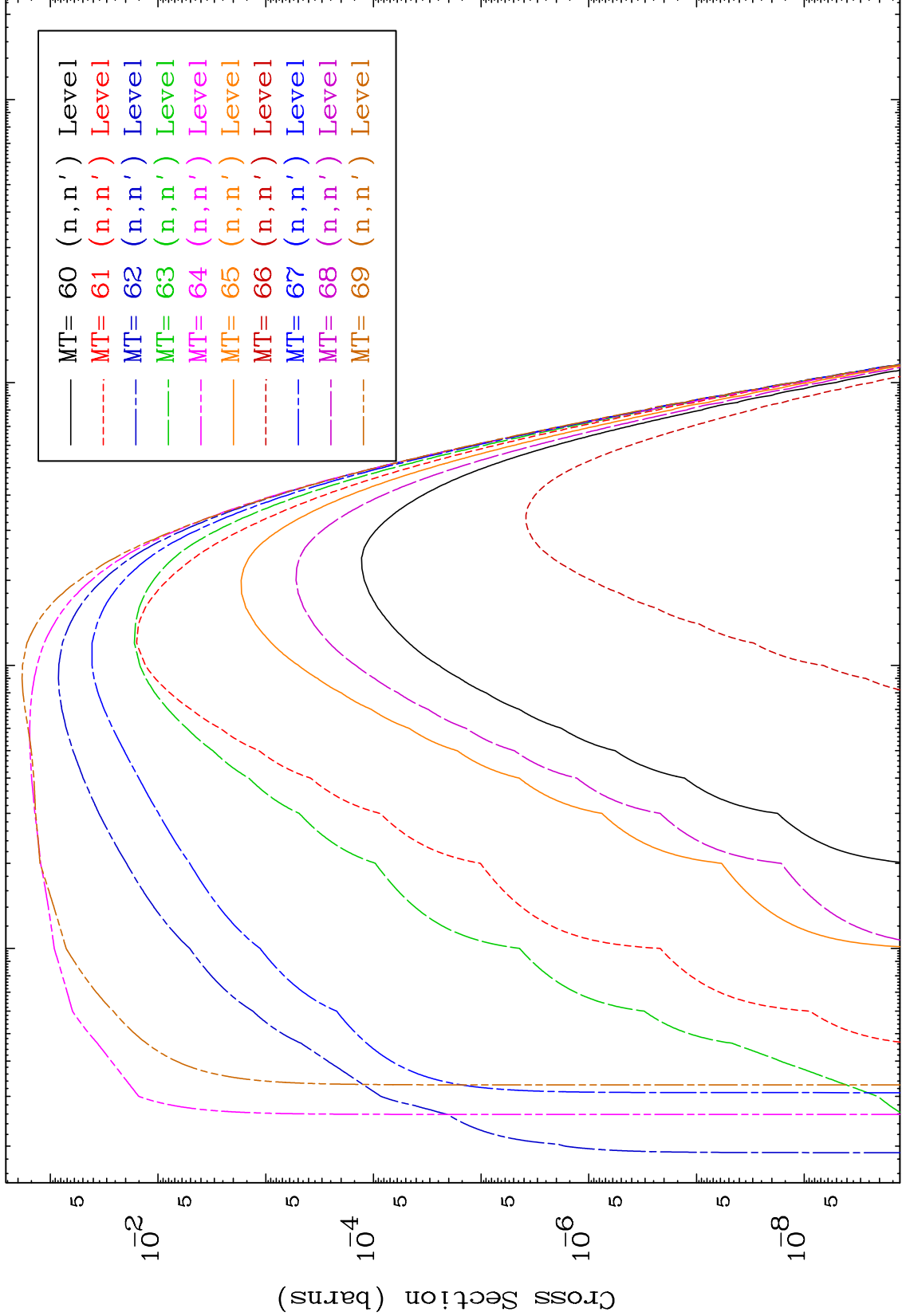


MAT 9936

(n,n') Level

100-Fm-250

293 Kelvin Cross Sections



9

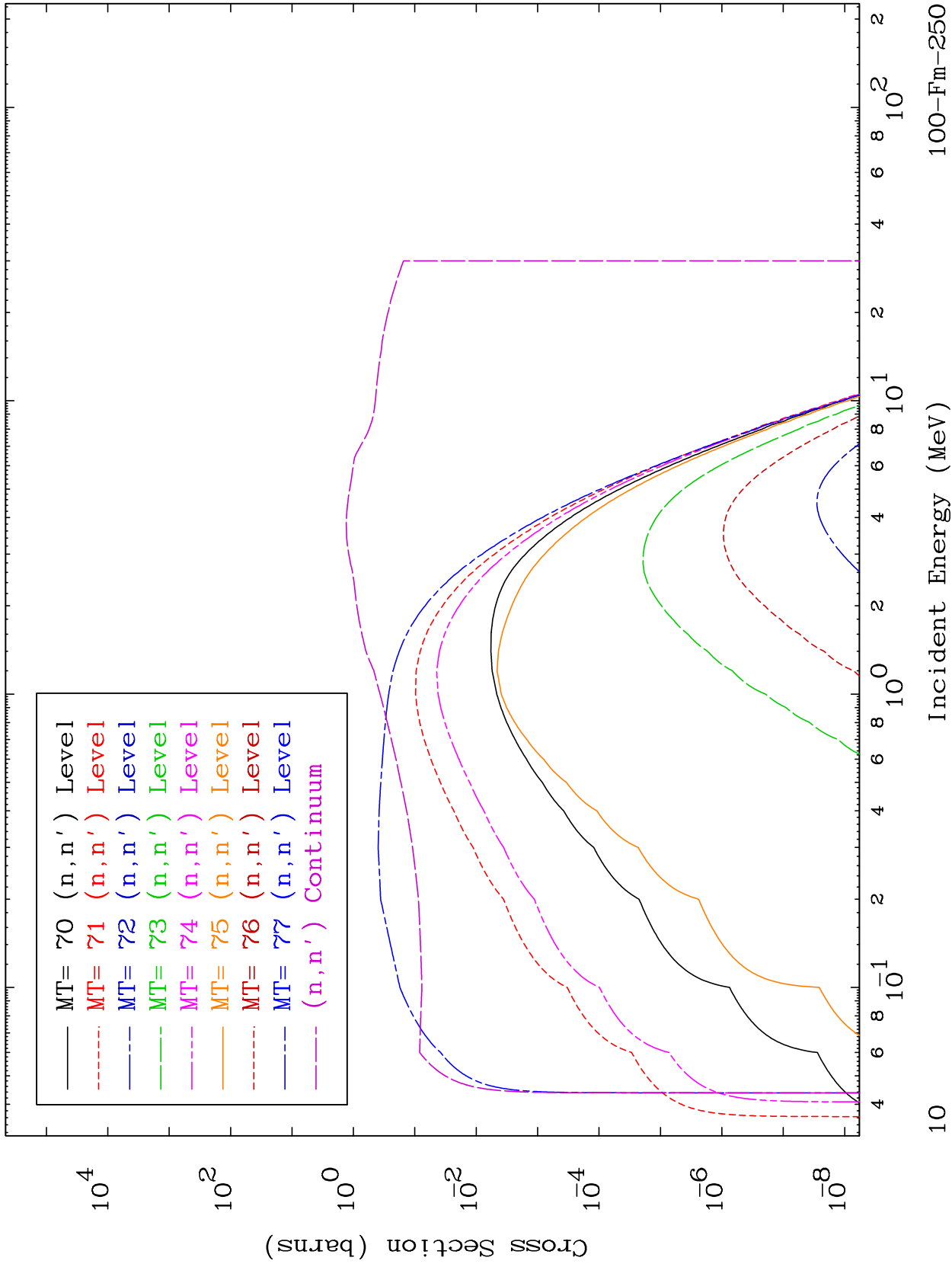
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,n') Level  
293 Kelvin Cross Sections

100-Fm-250



10

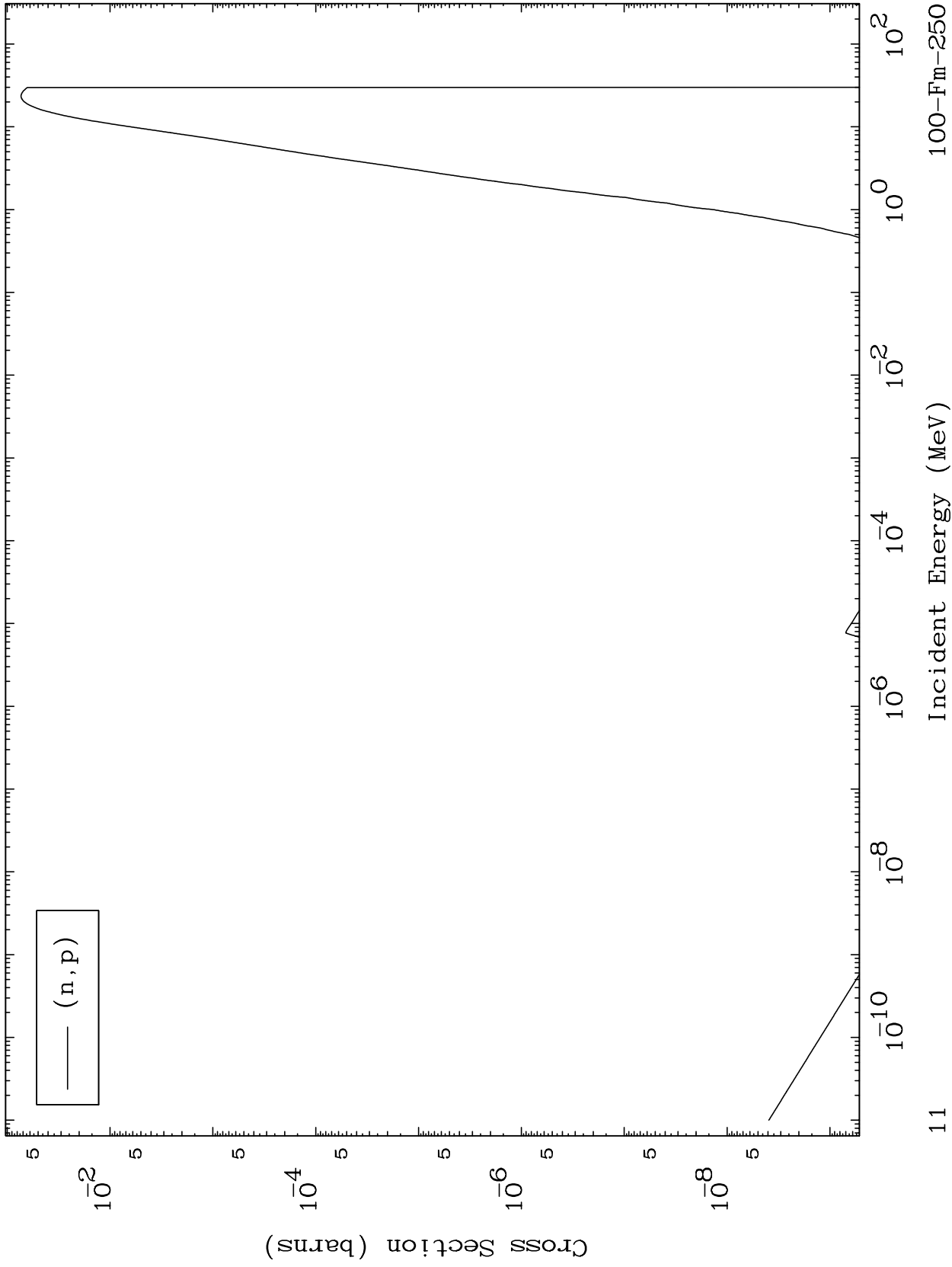
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,p) Levels  
293 Kelvin Cross Sections

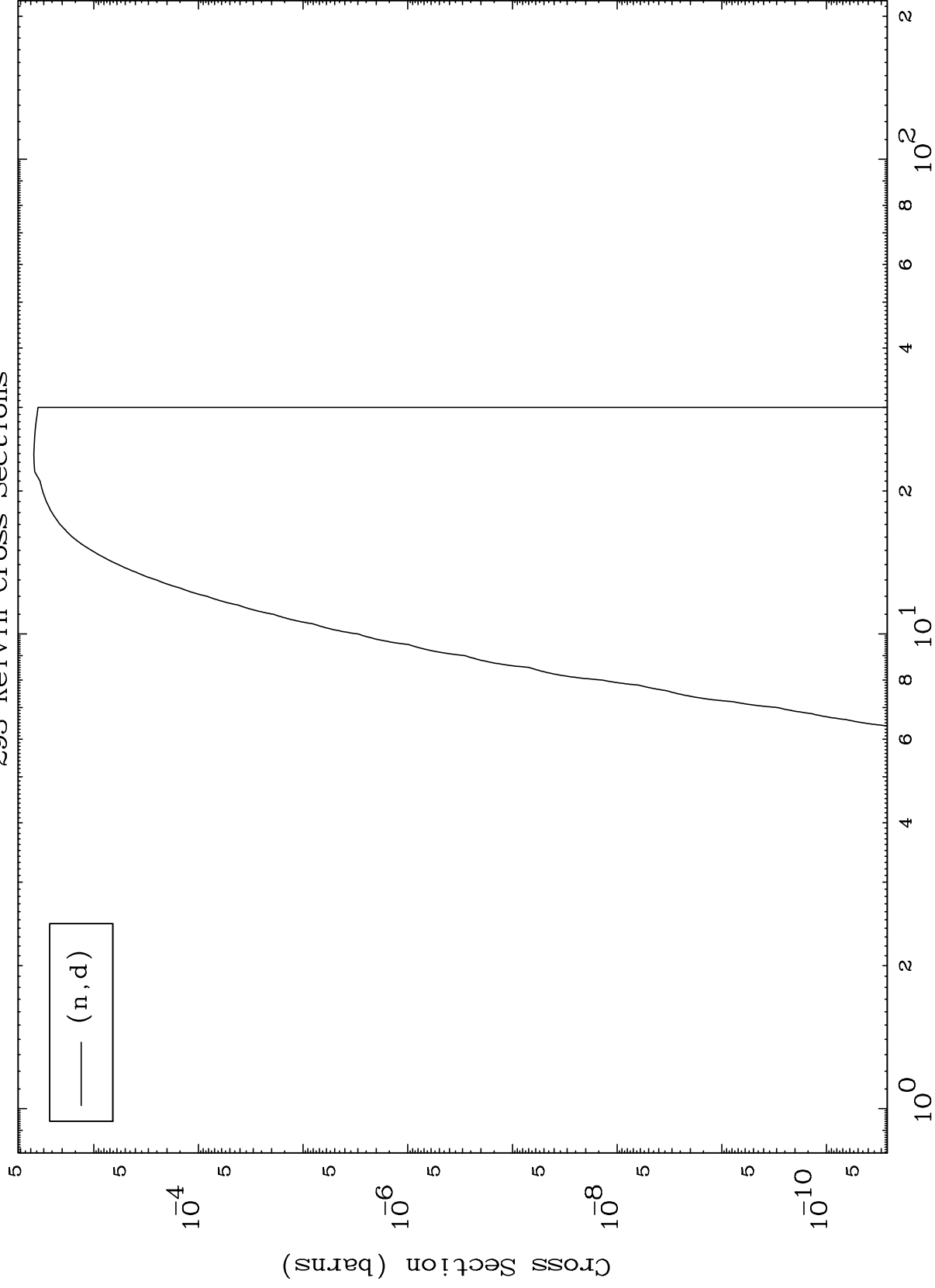
100-Fm-250



MAT 9936

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-250



12

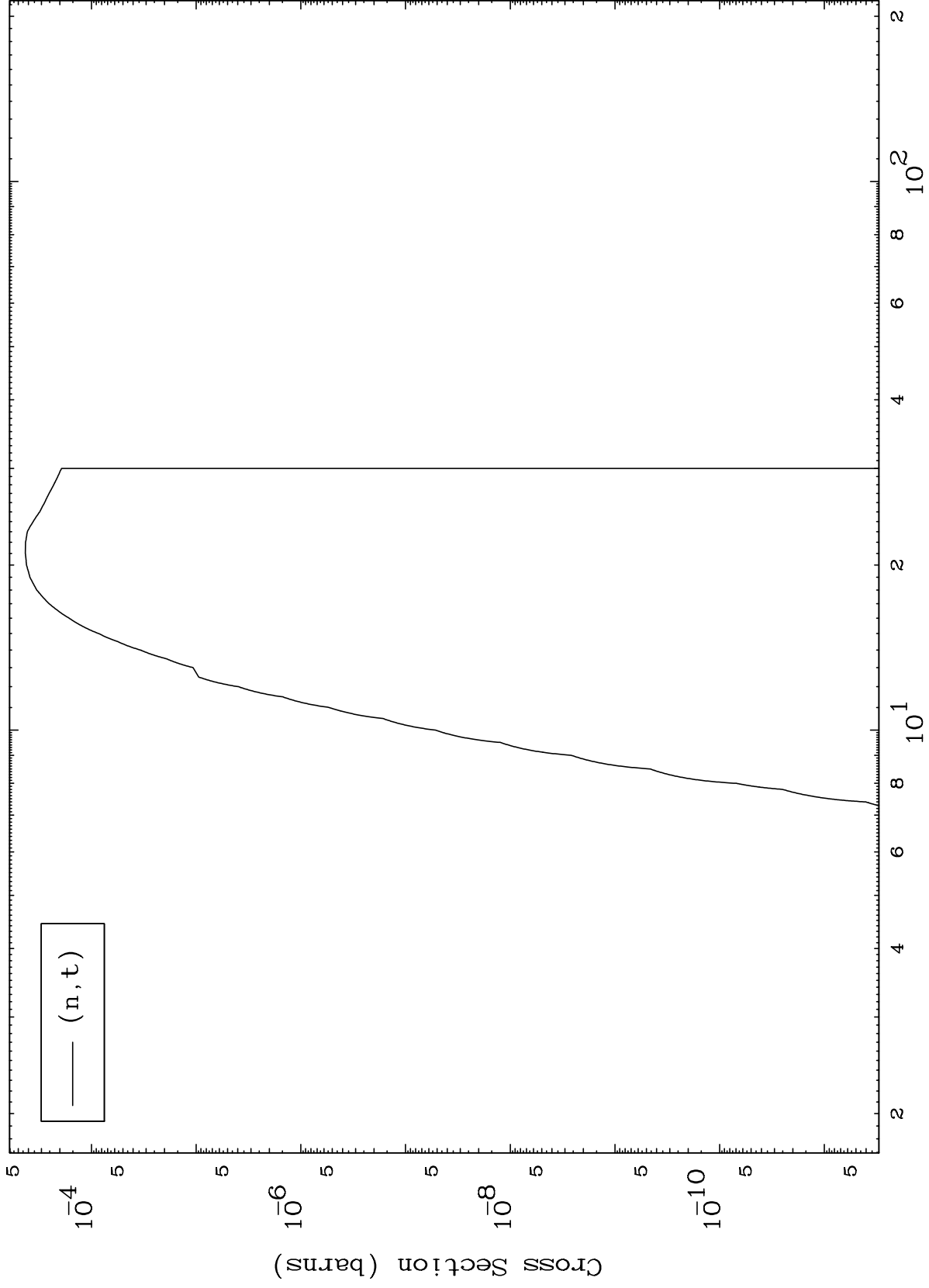
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-250



13

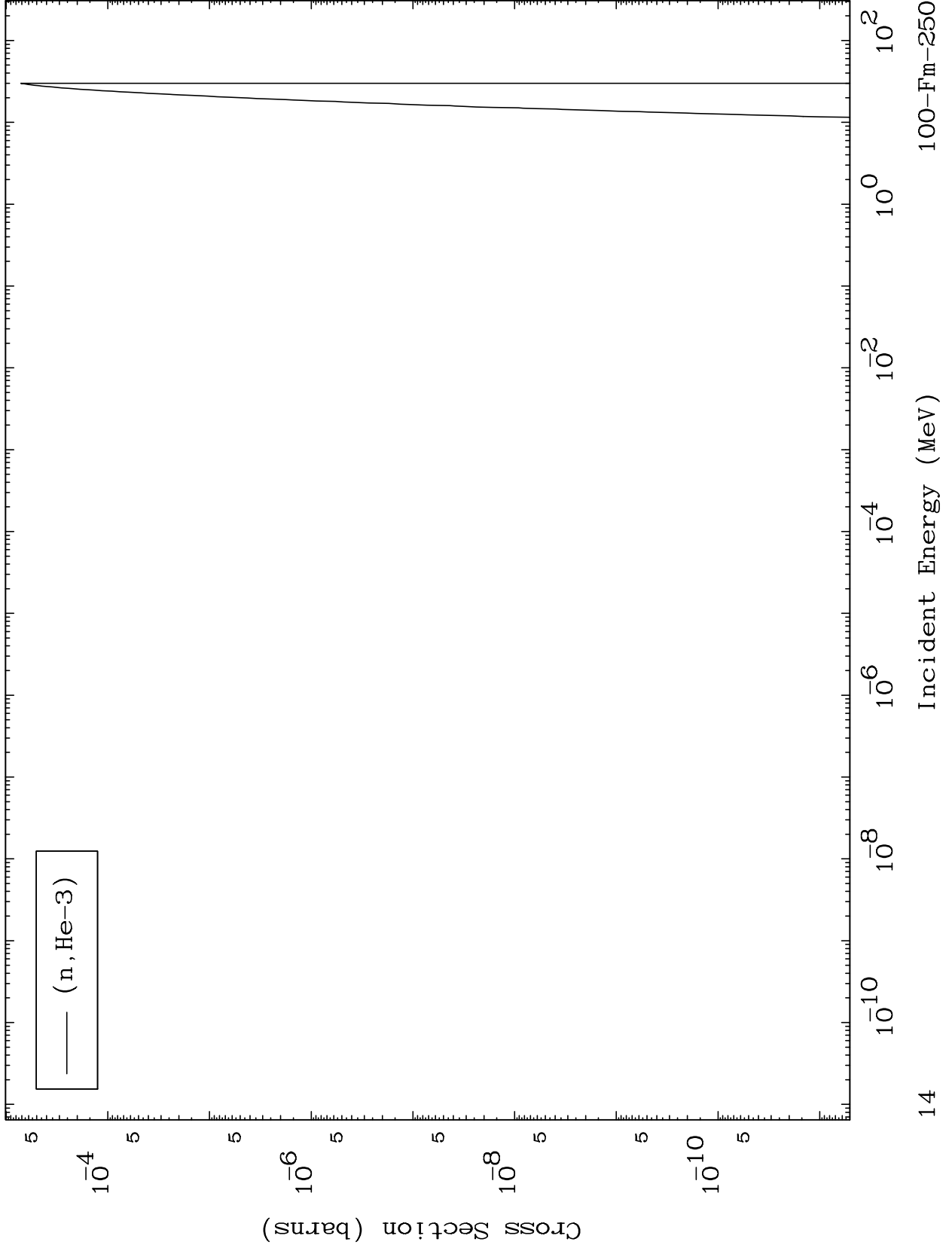
Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,He3) Levels  
293 Kelvin Cross Sections

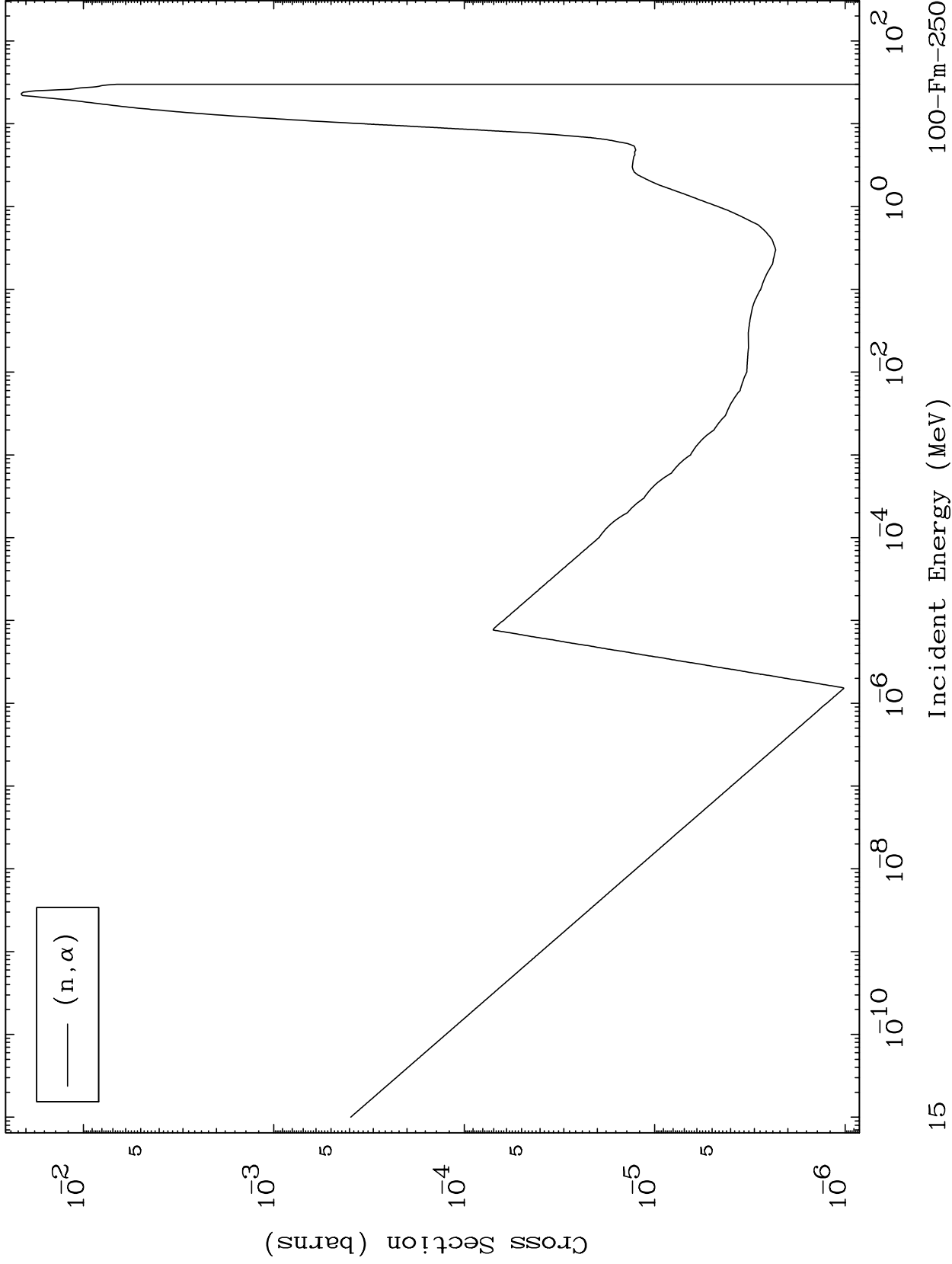
100-Fm-250



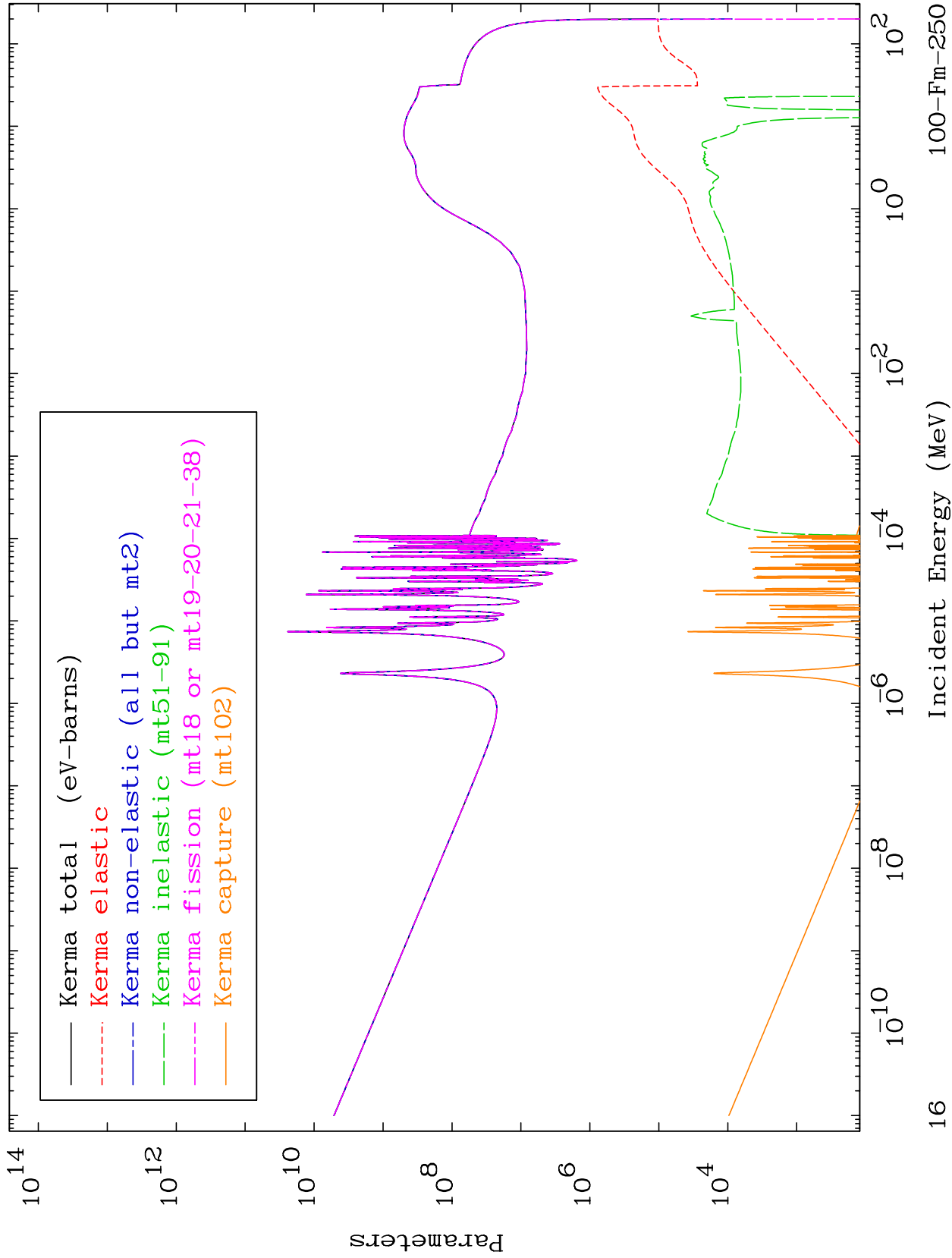
MAT 9936

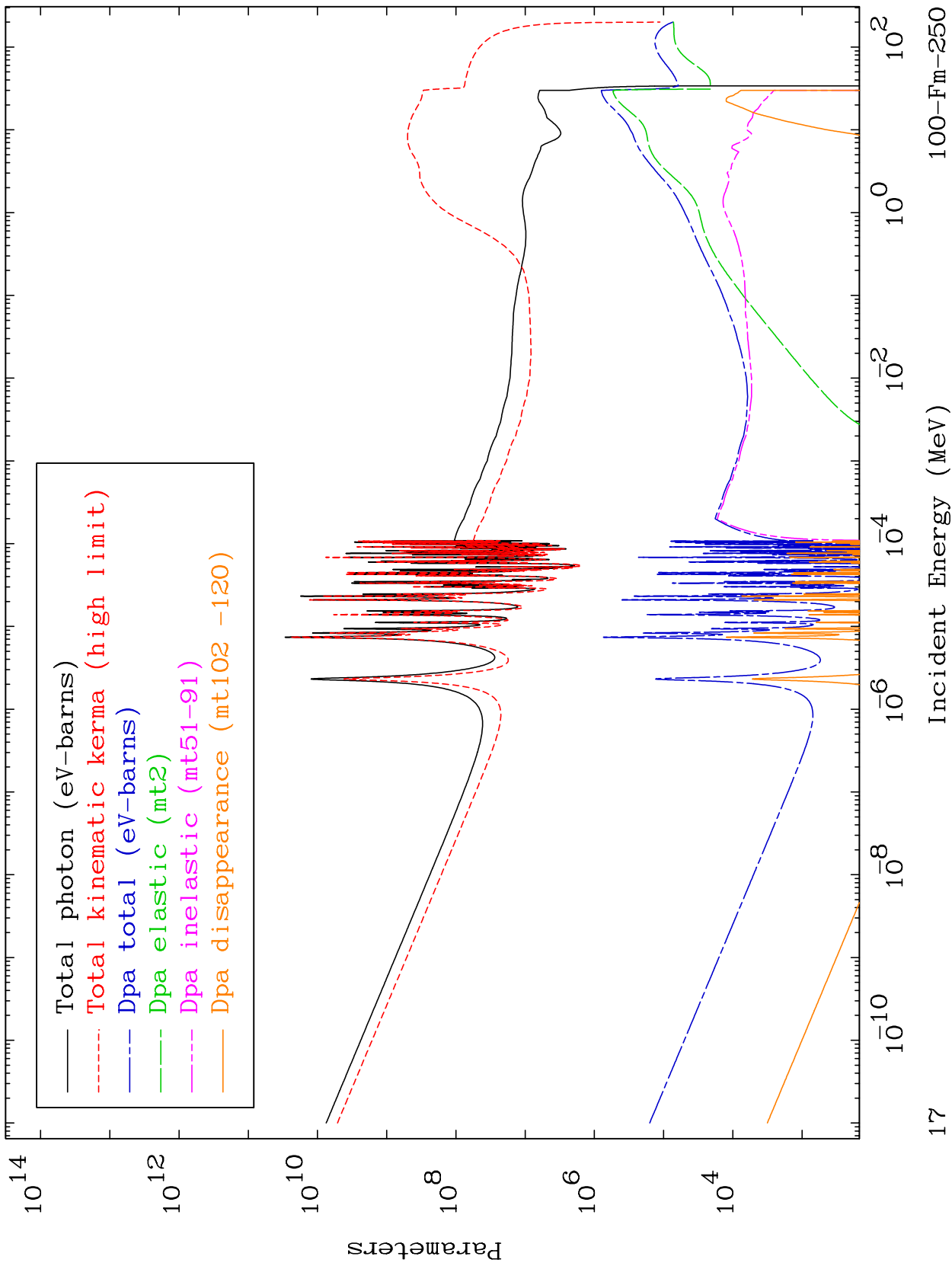
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

100-Fm-250





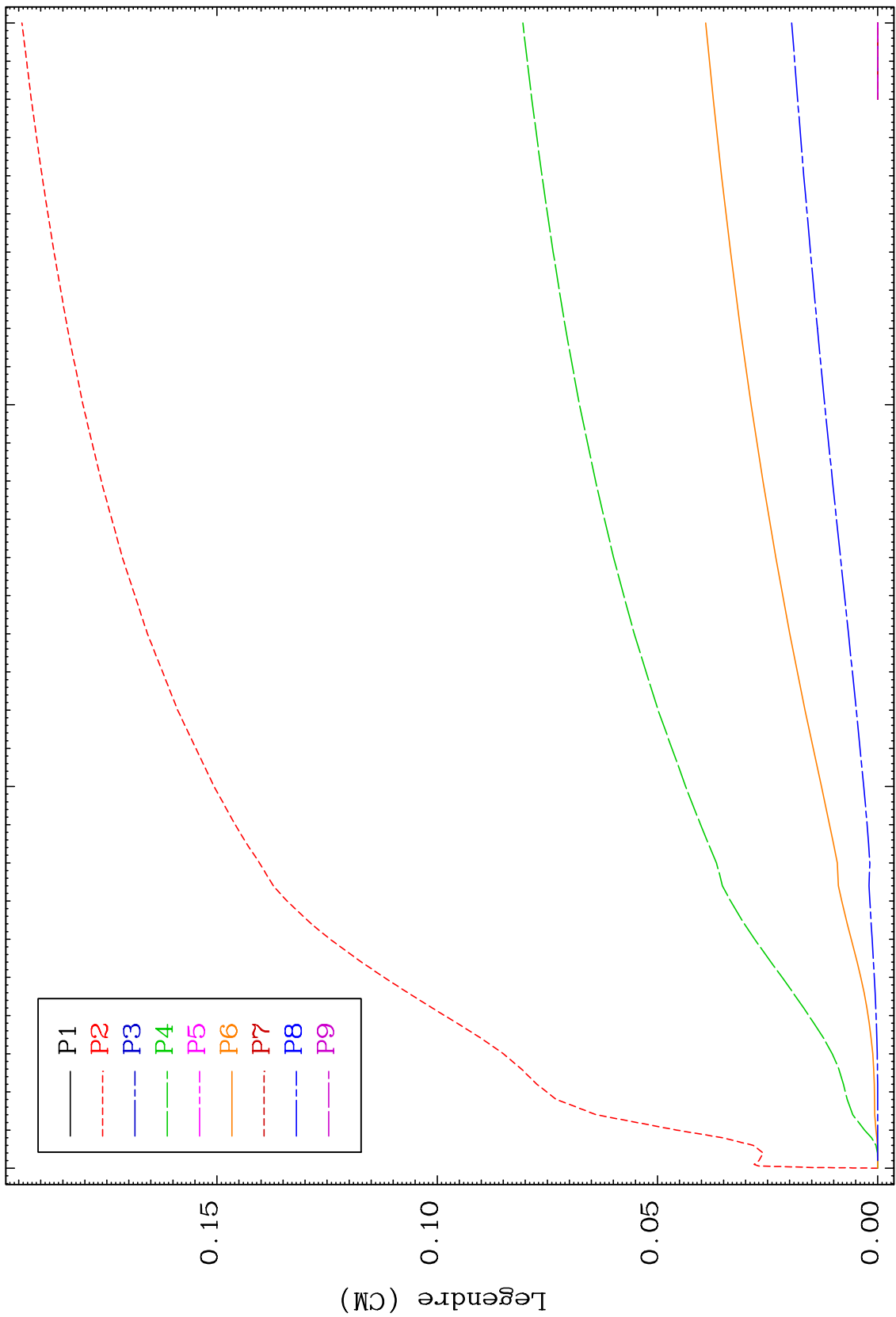




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Elastic Legendre Coefficients

100-Fm-250



18

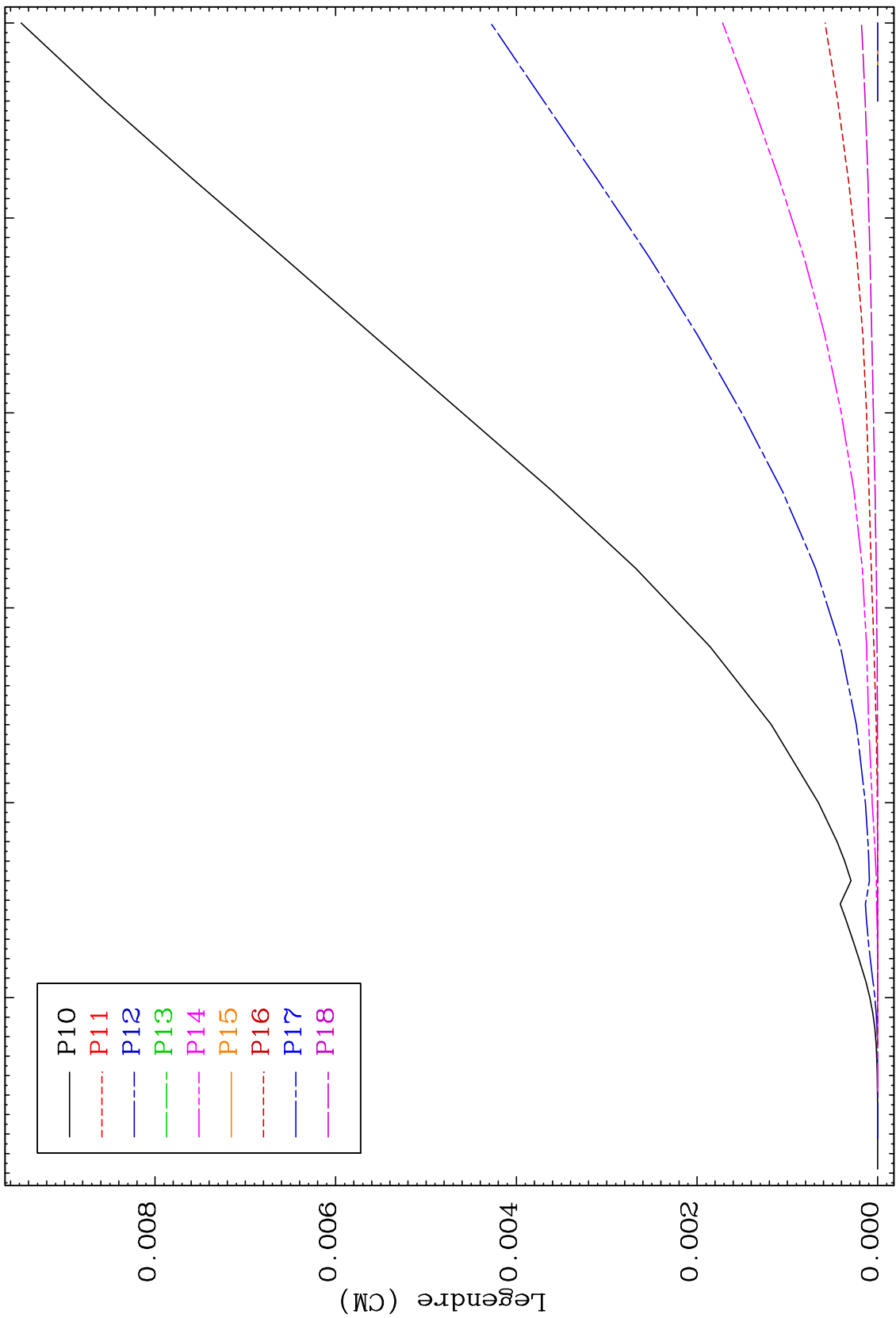
Incident Energy (MeV)

100-Fm-250

MAT 9936

Elastic Legendre Coefficients

100-Fm-250



19

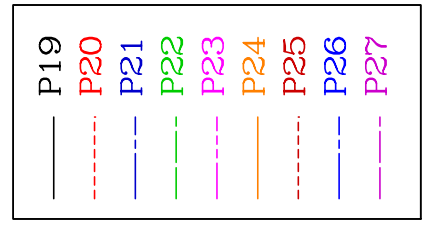
Incident Energy (MeV)

100-Fm-250

MAT 9936

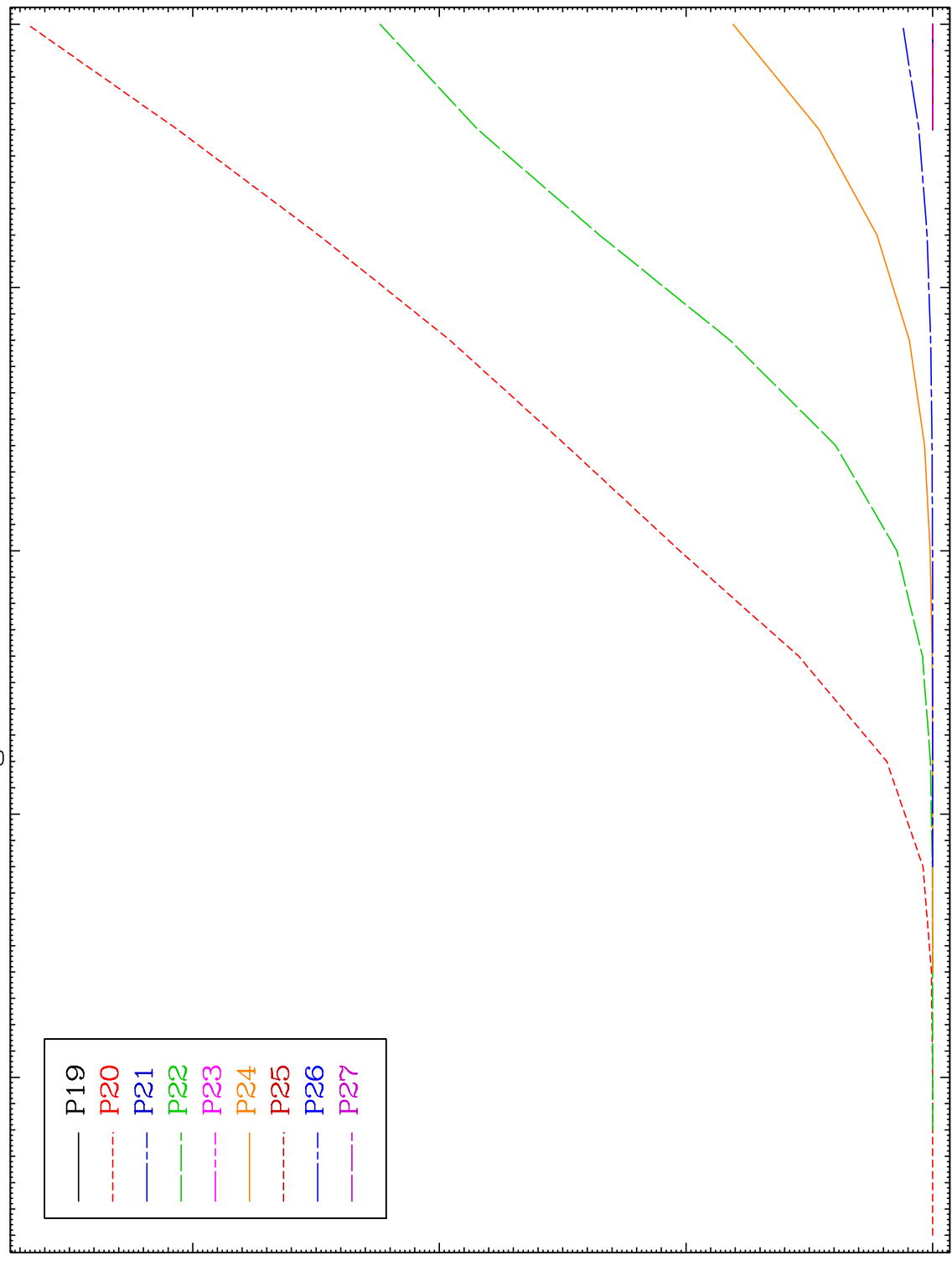
Elastic  
Legendre Coefficients

100-Fm-250



$\times 10^{-5}$

Legendre (CM)



20

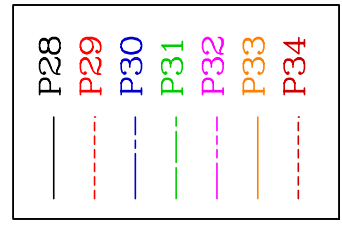
Incident Energy (MeV)

100-Fm-250

MAT 9936

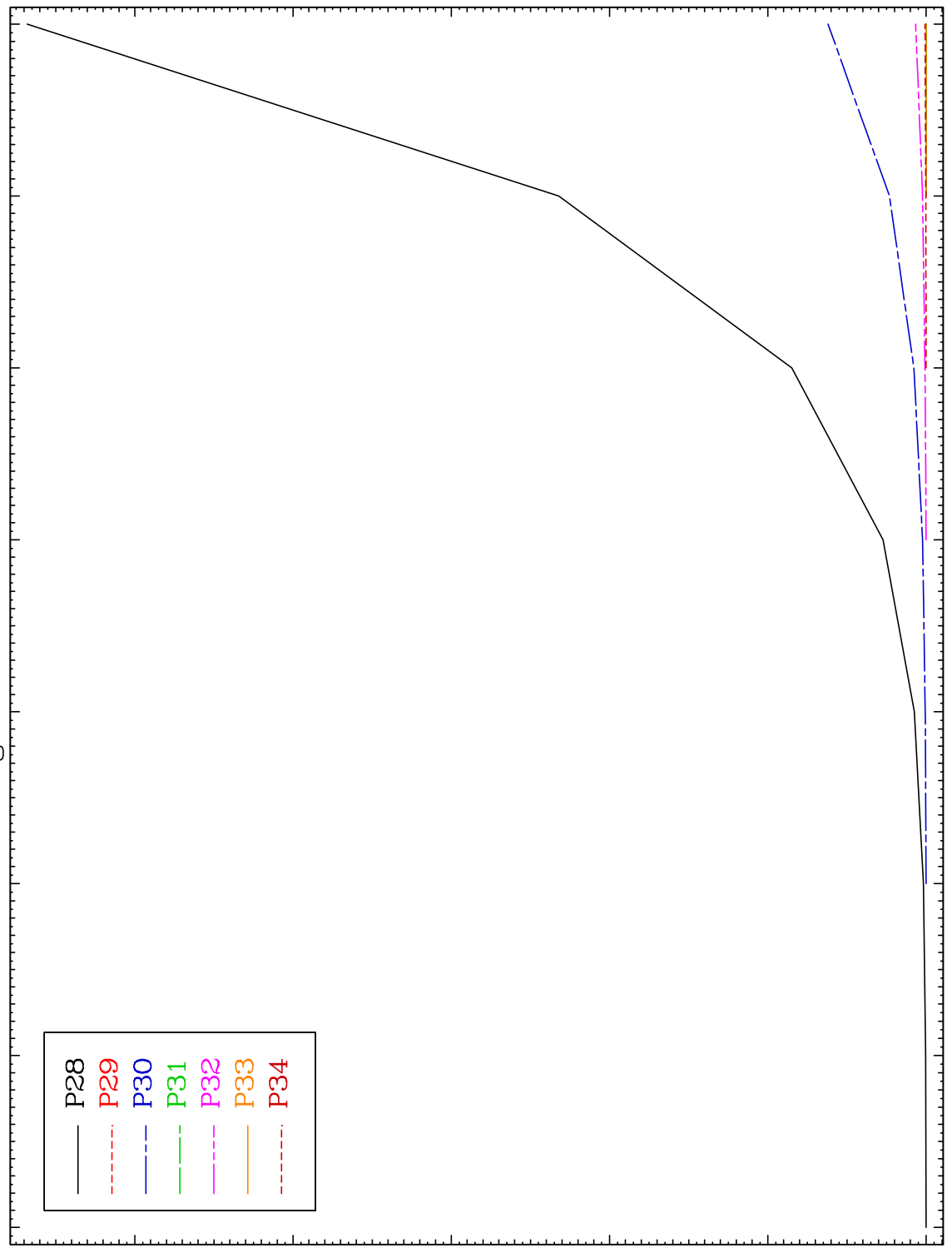
Elastic Legendre Coefficients

100-Fm-250



$\times 10^{-7}$

Legendre (CM)



21

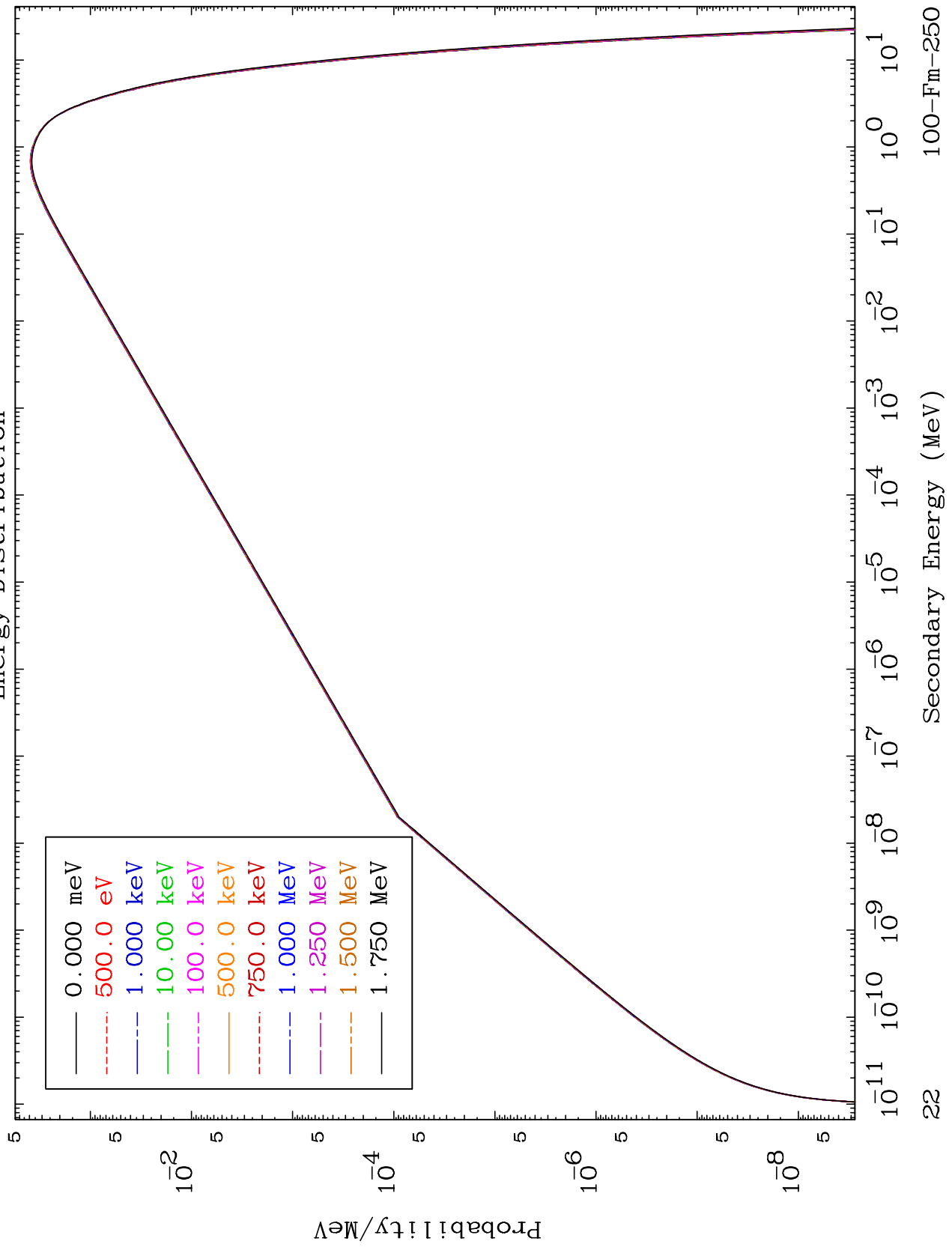
Incident Energy (MeV)

100-Fm-250

MAT 9936

### Fission Energy Distribution

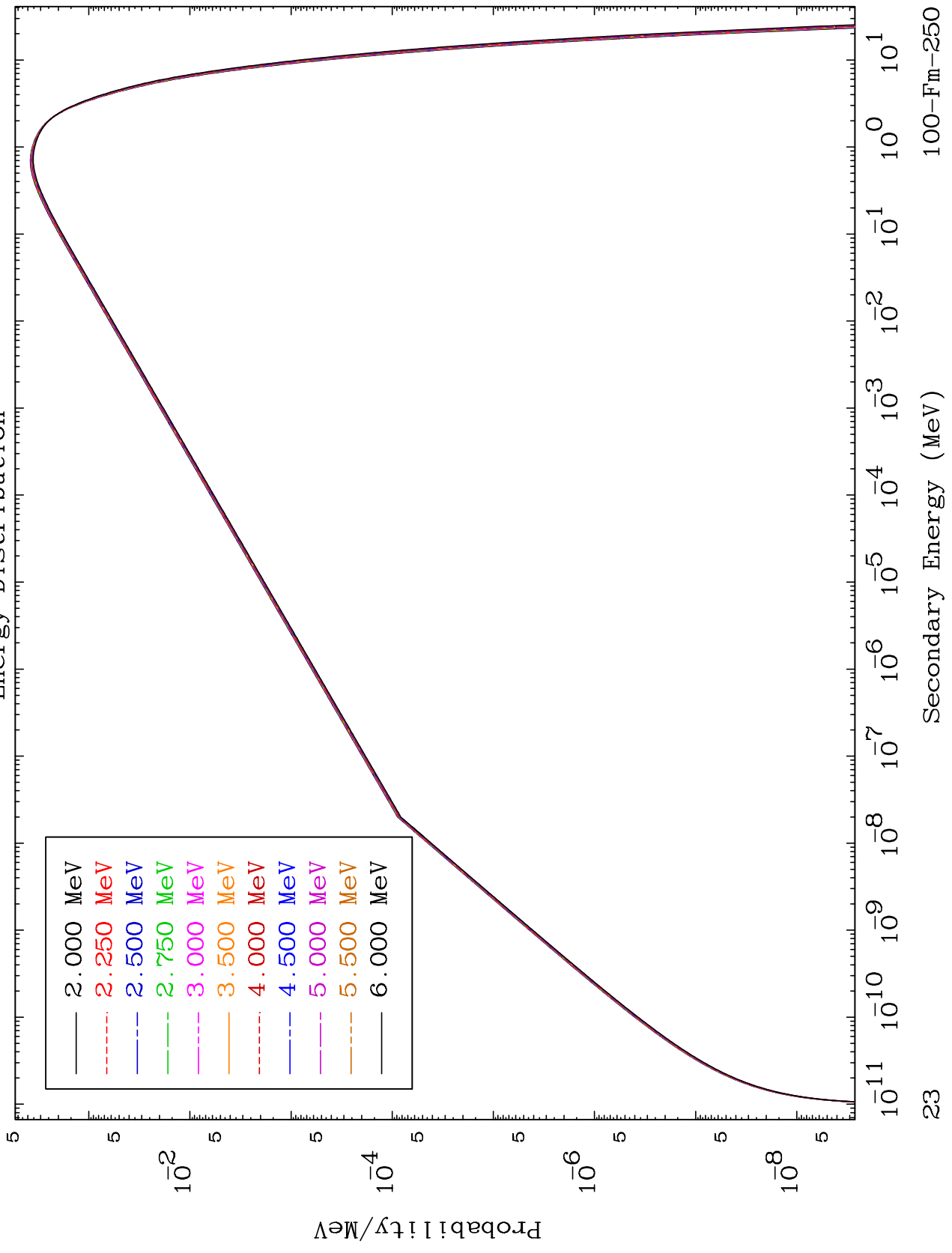
100-Fm-250



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### Fission Energy Distribution

100-Fm-250

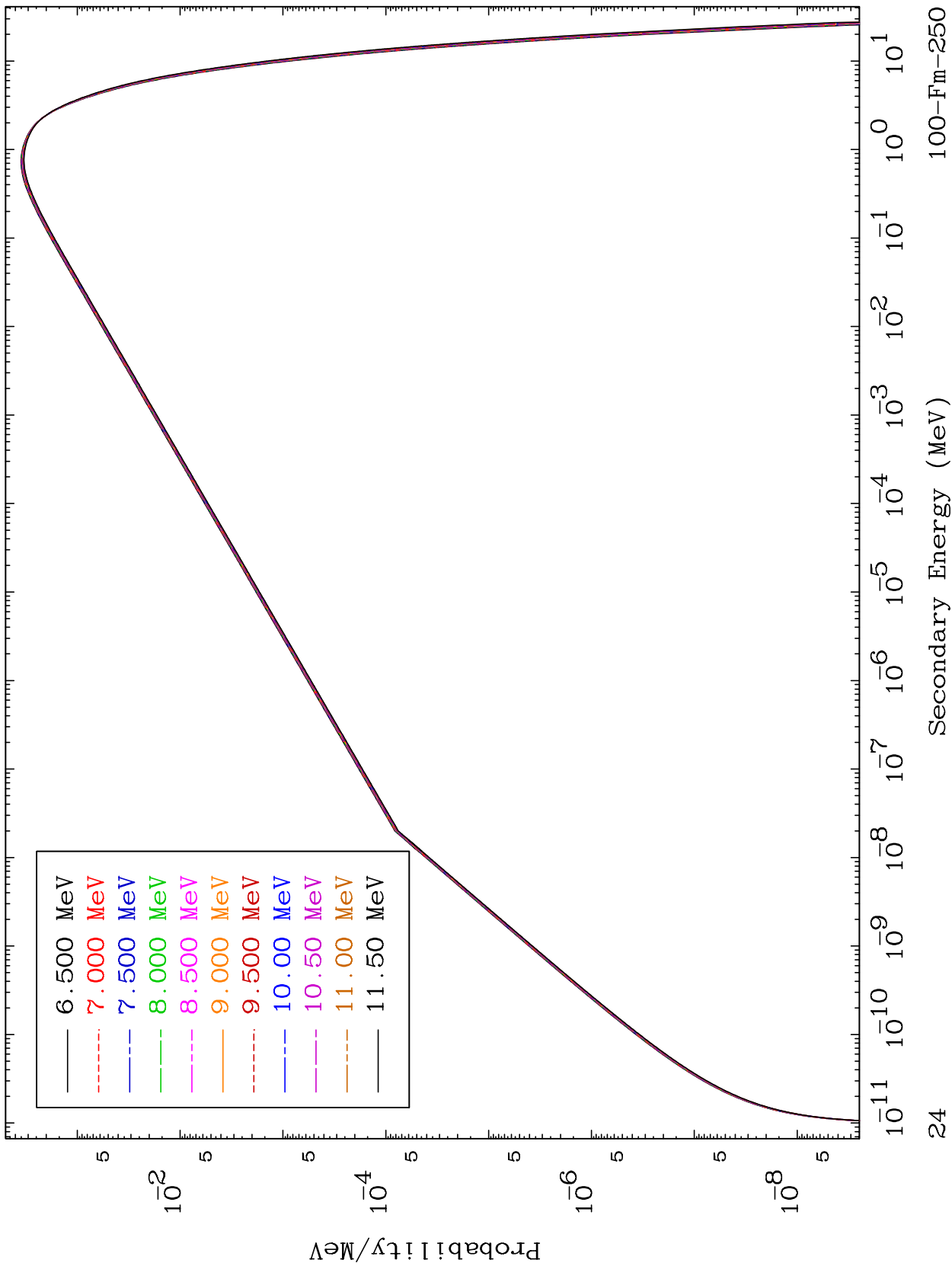




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### Fission Energy Distribution

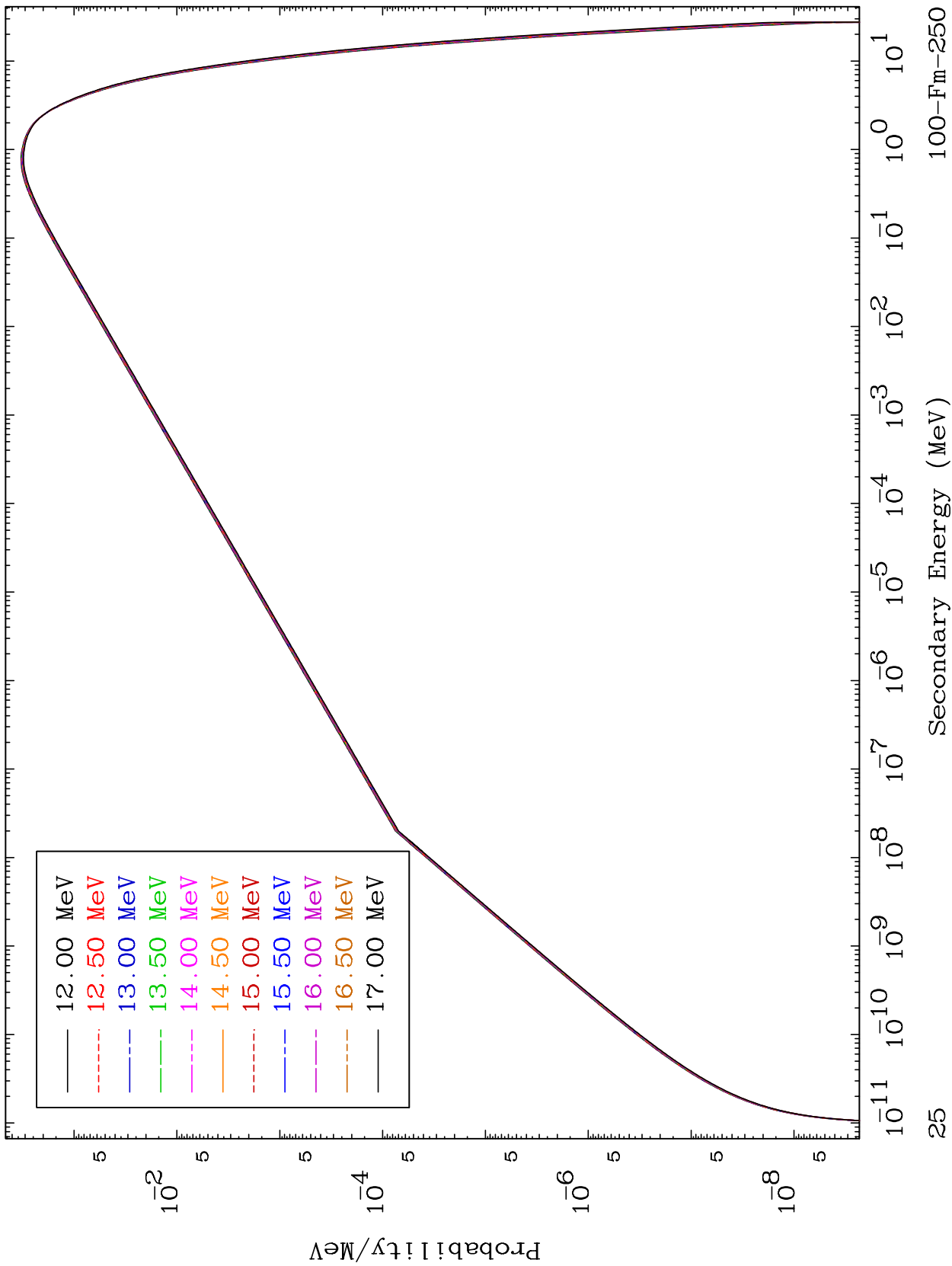
100-Fm-250



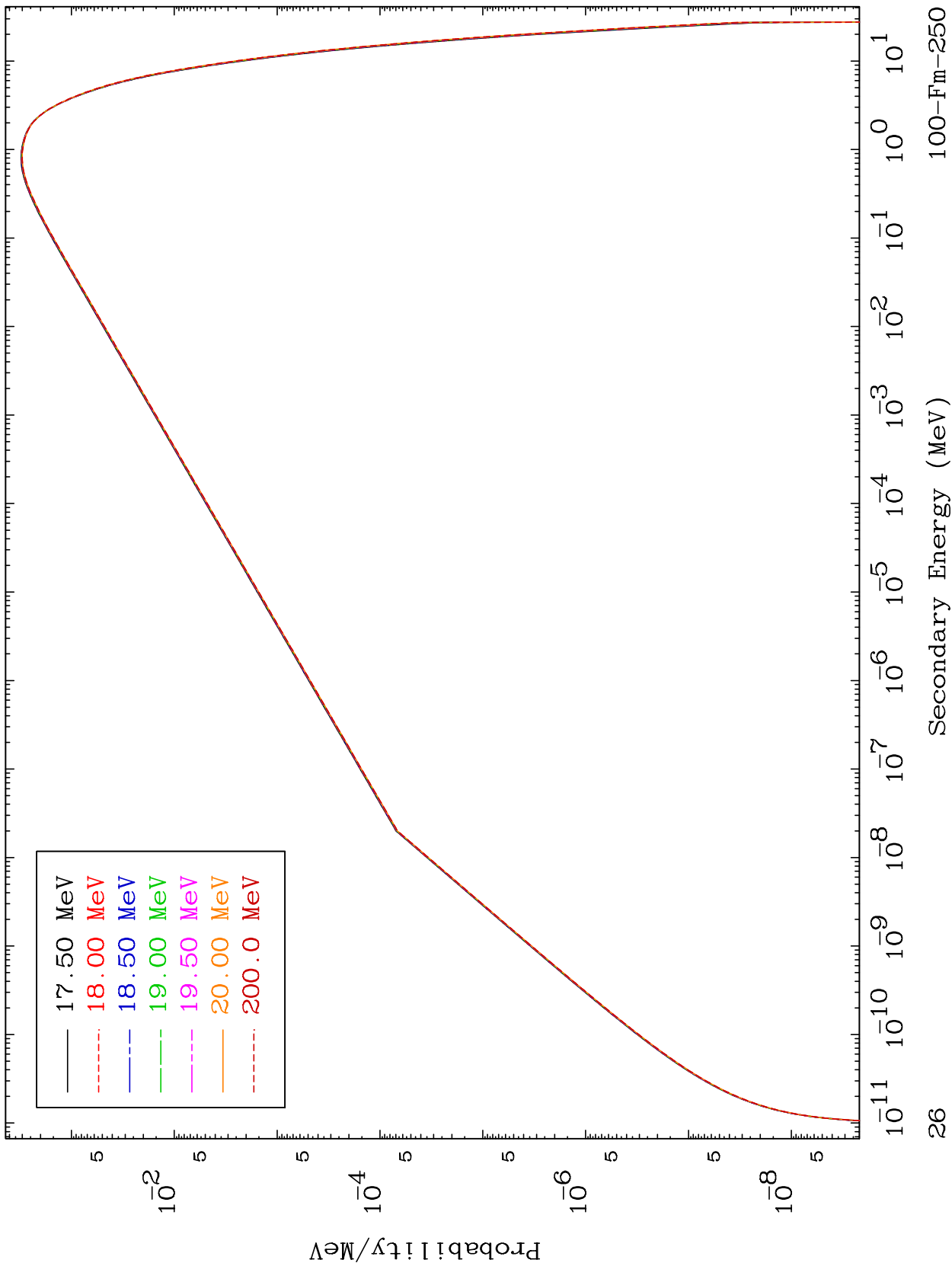
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Fission Energy Distribution

100-Fm-250



### Fission Energy Distribution



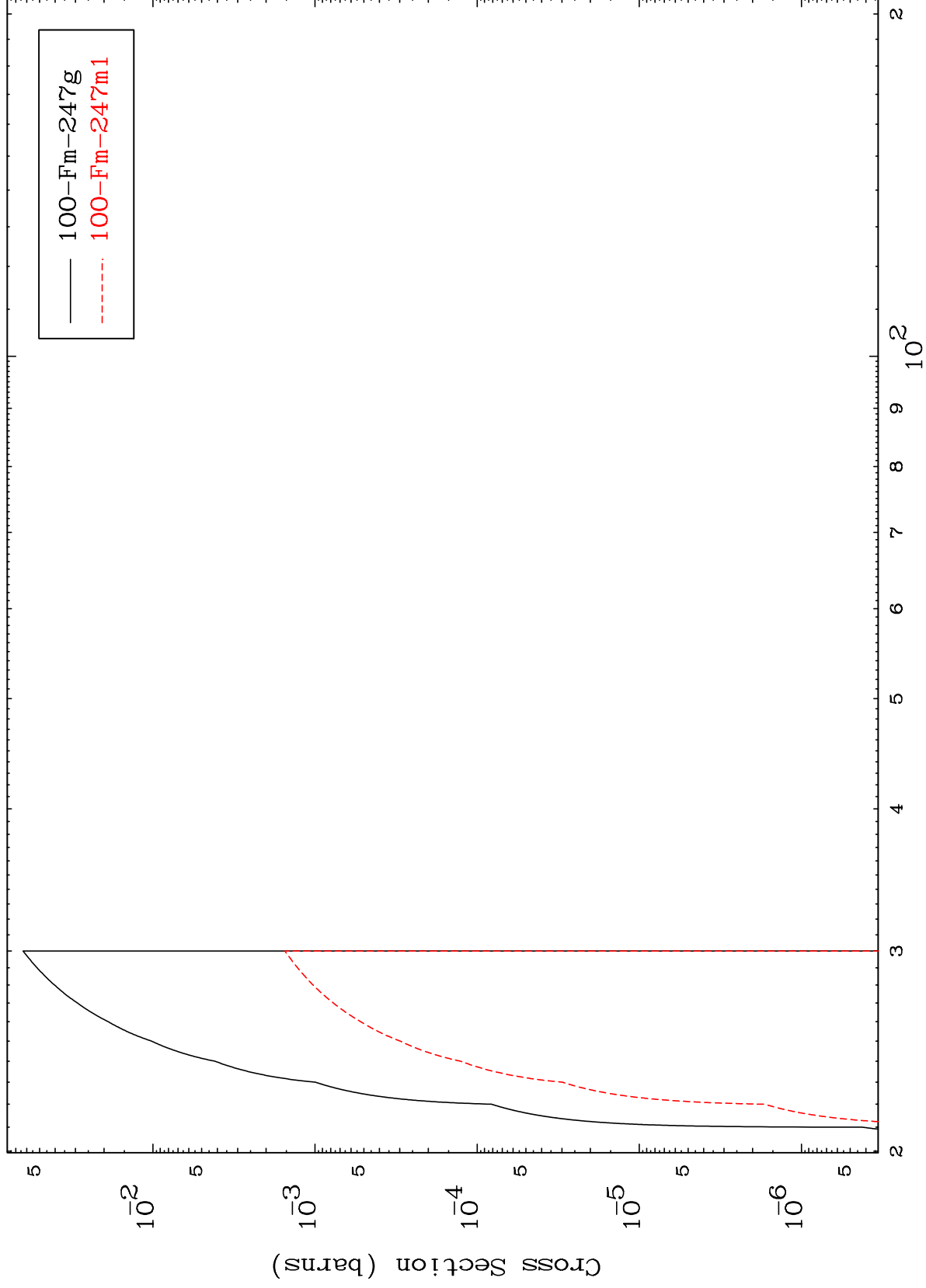


MAT 9936

(n,4n)

100-Fm-250

Radionuclide Production Cross Section



28

Incident Energy (MeV)

100-Fm-250

MAT 9936

(n,p)

100-Fm-250

Radionuclide Production Cross Section

