

Program EVALPLOT  
(Version 2018-1)

by

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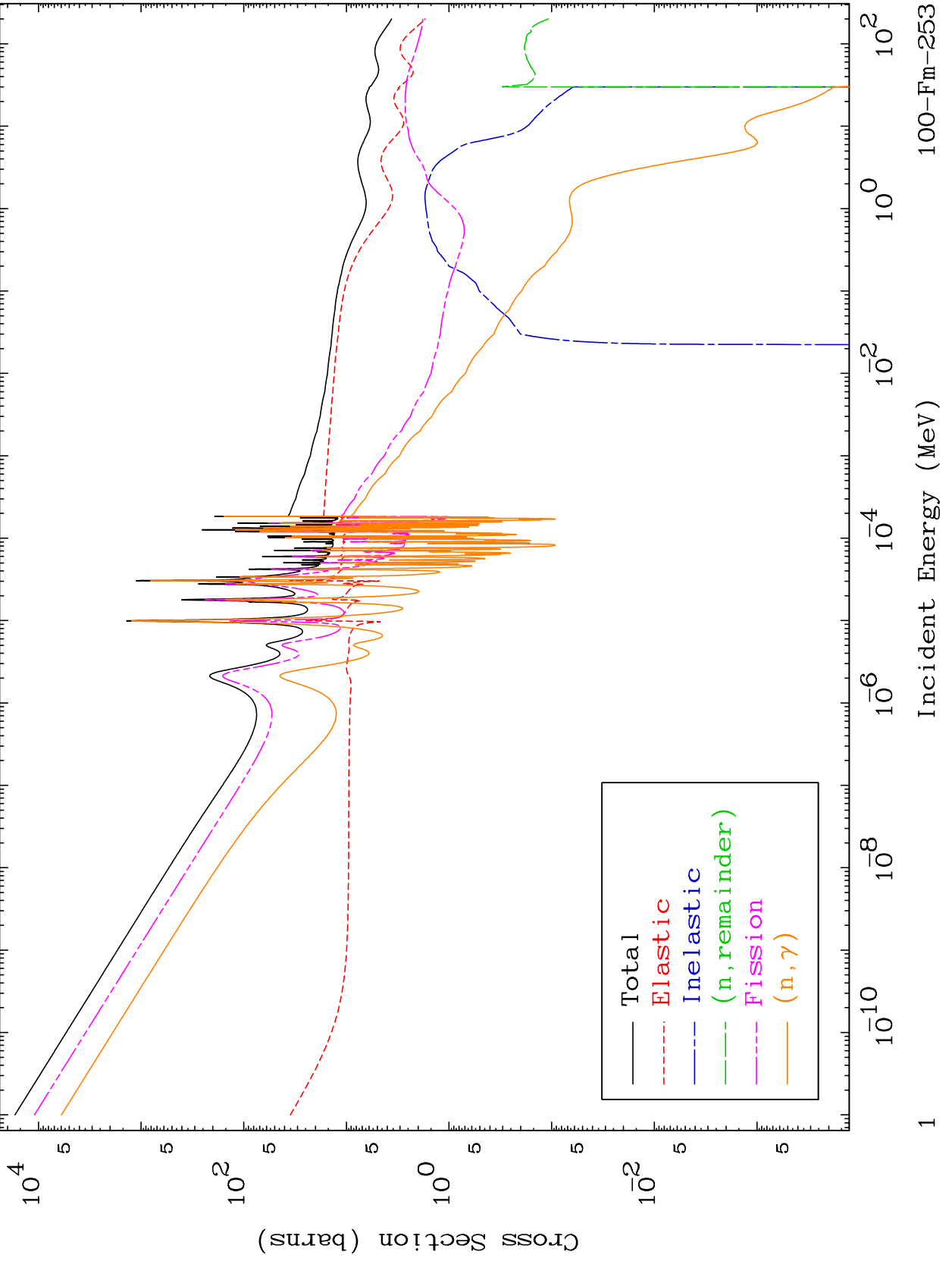
Press Mouse Button to Start

MAT 9934

Major

293 Kelvin Cross Sections

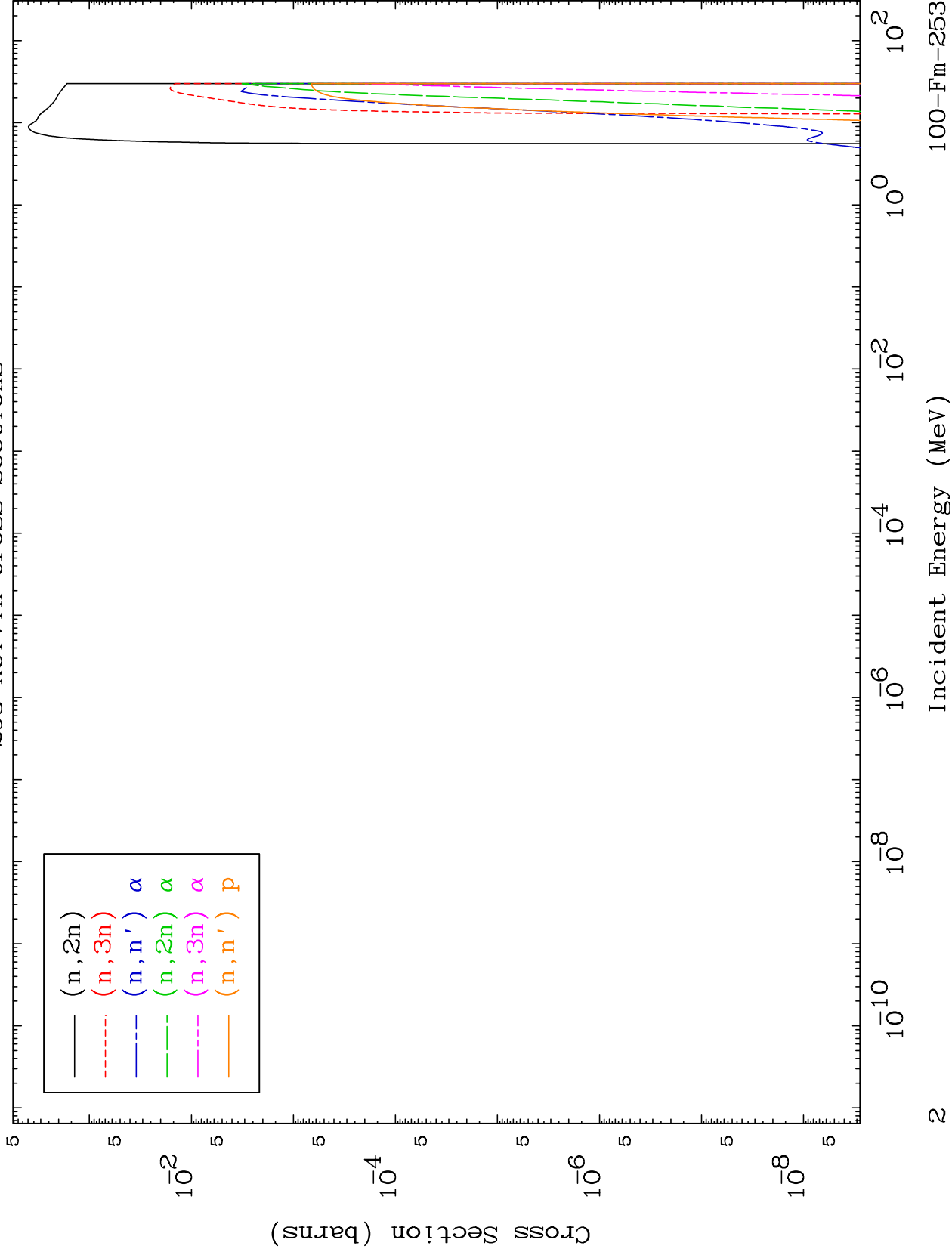
100-Fm-253



MAT 9934

Neutron Production  
293 Kelvin Cross Sections

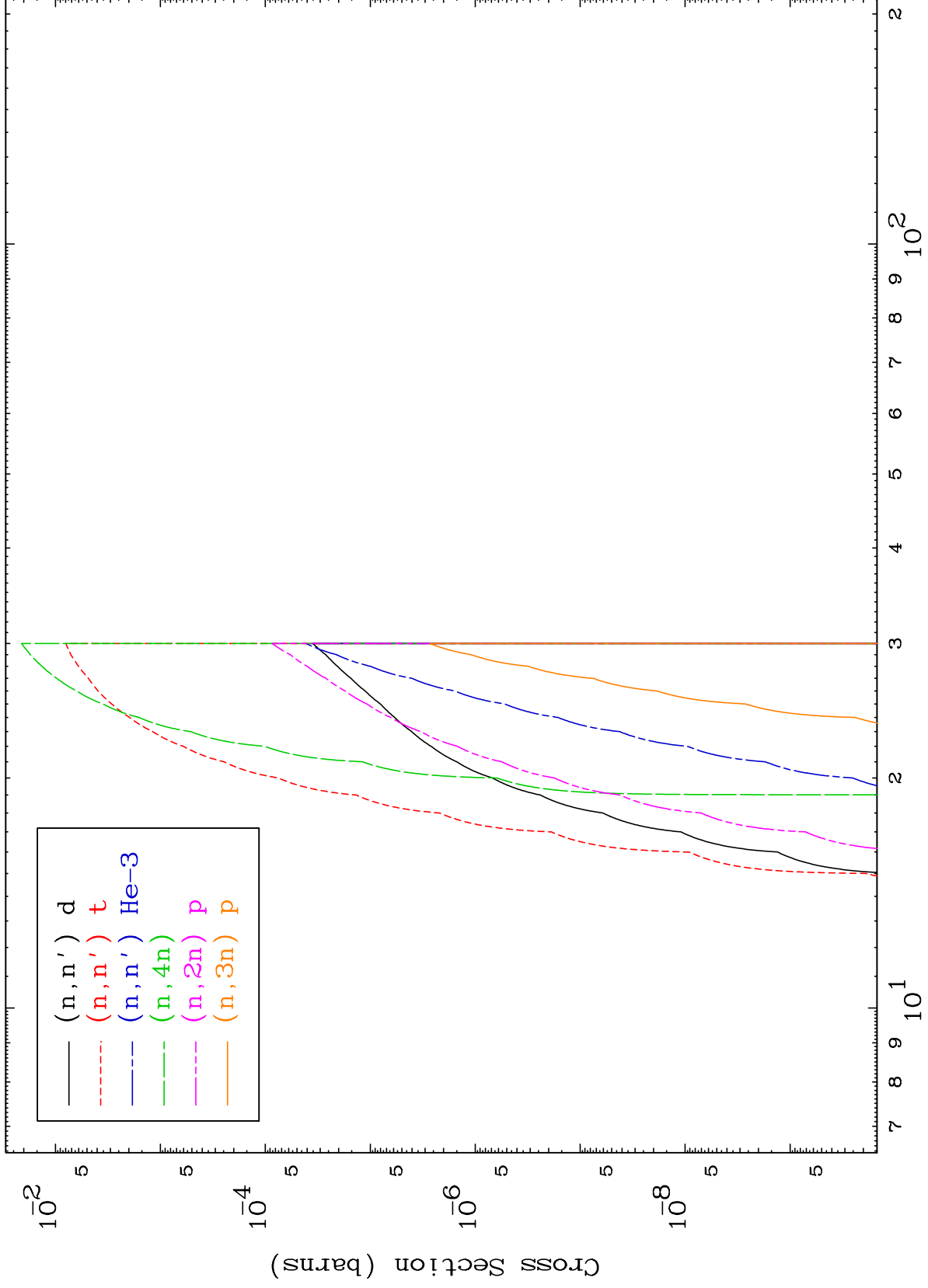
100-Fm-253



MAT 9934

Neutron Production  
293 Kelvin Cross Sections

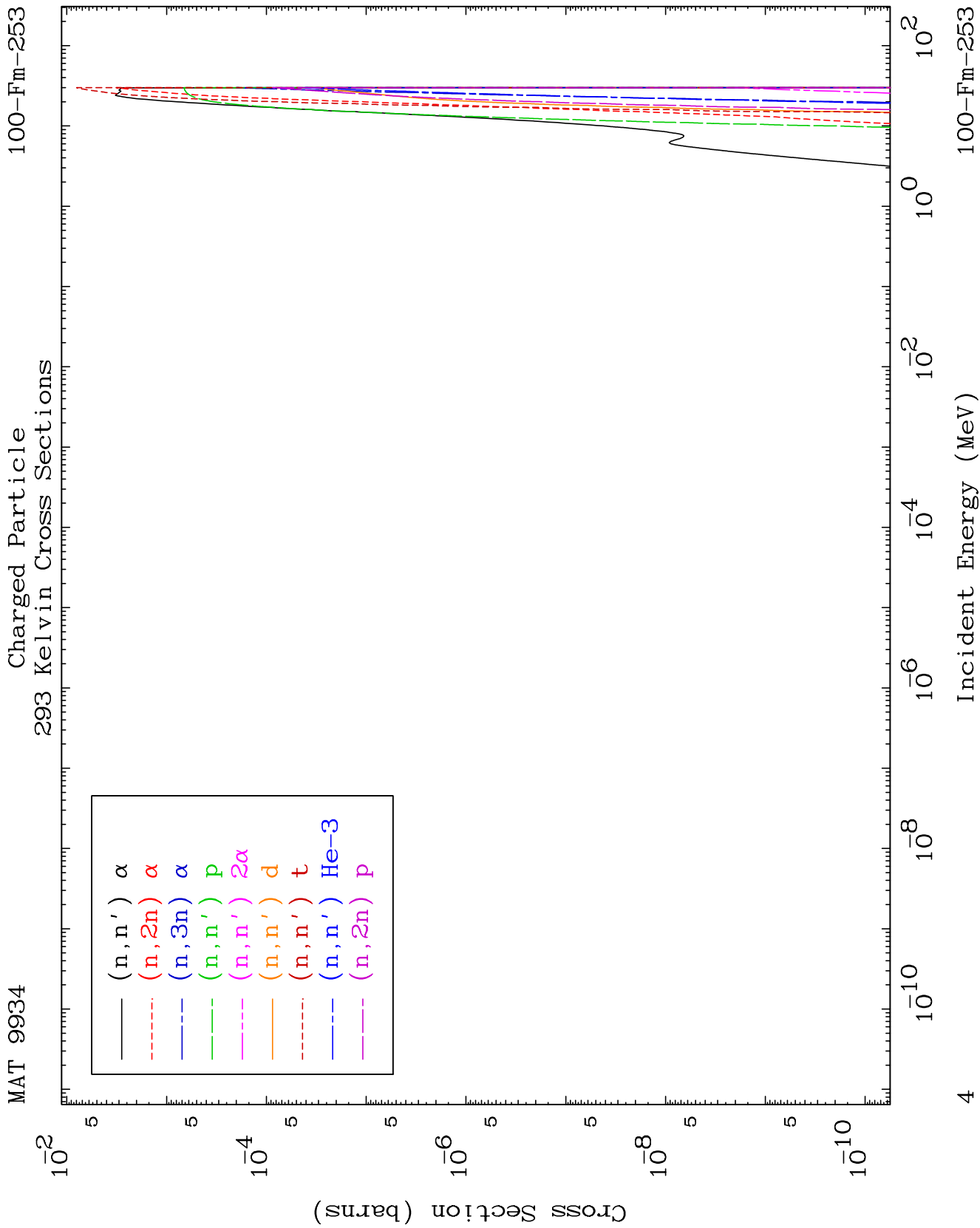
100-Fm-253



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Incident Energy (MeV)

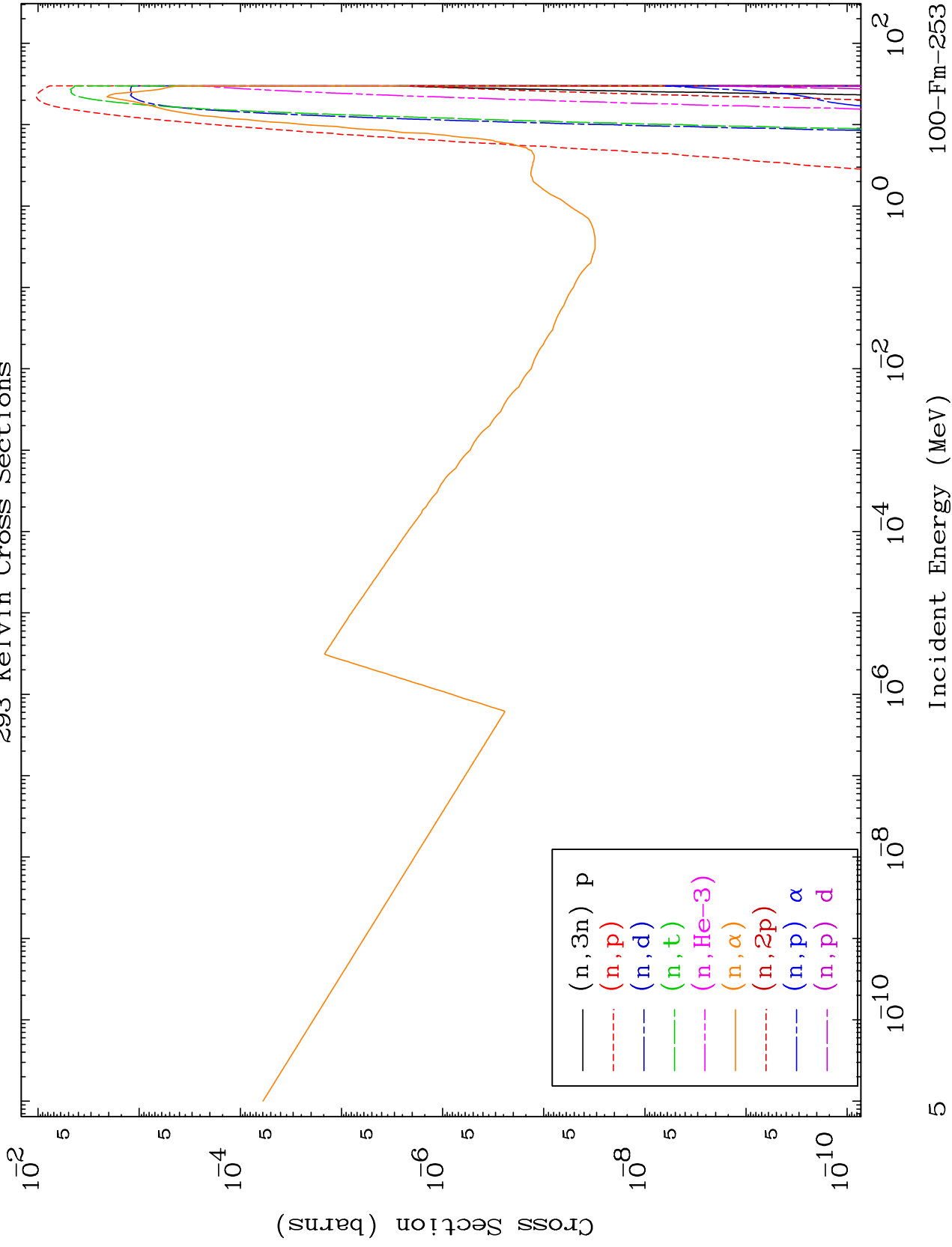
100-Fm-253



MAT 9934

Charged Particle  
293 Kelvin Cross Sections

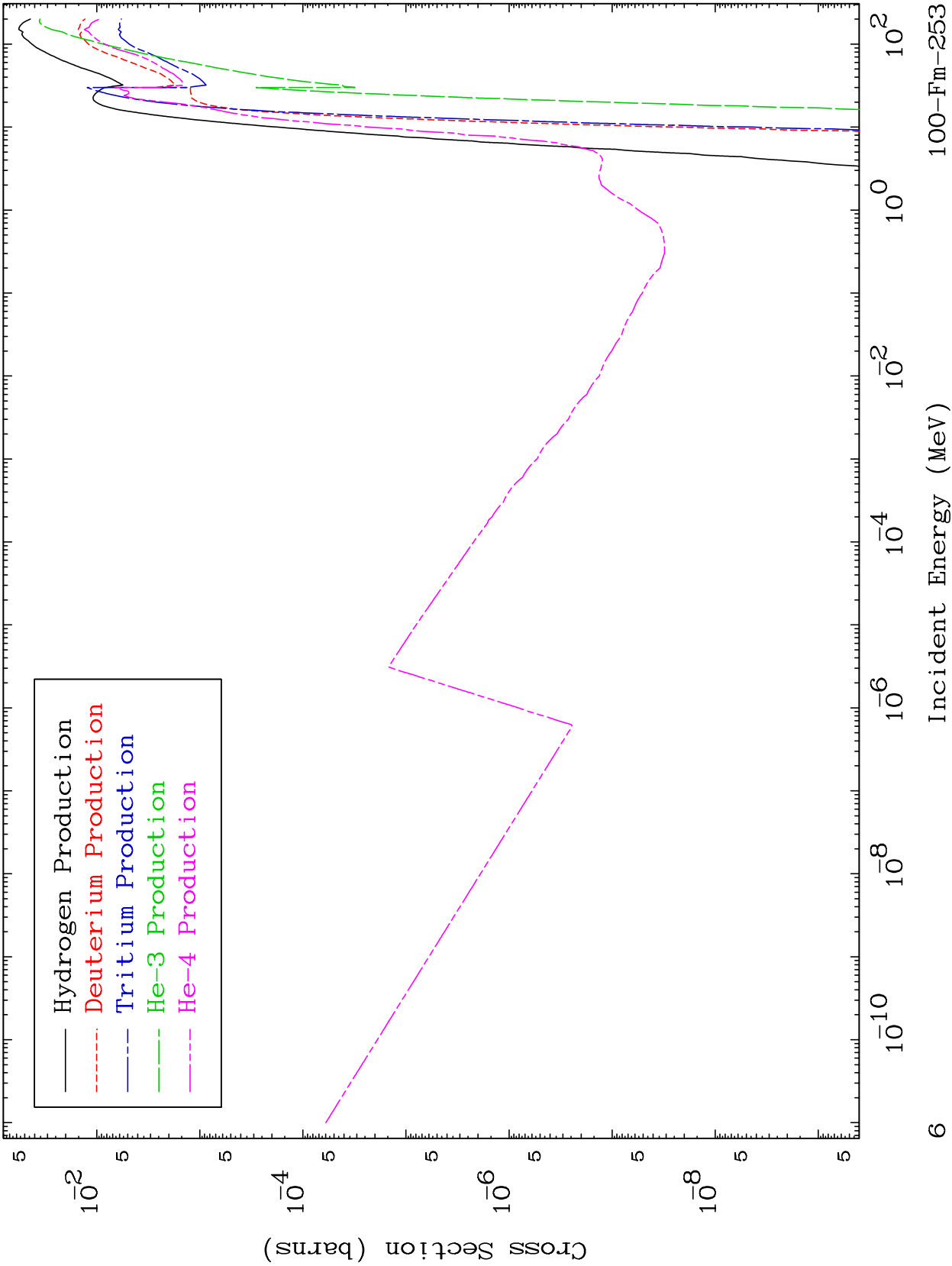
100-Fm-253



MAT 9934

Particle Production  
293 Kelvin Cross Sections

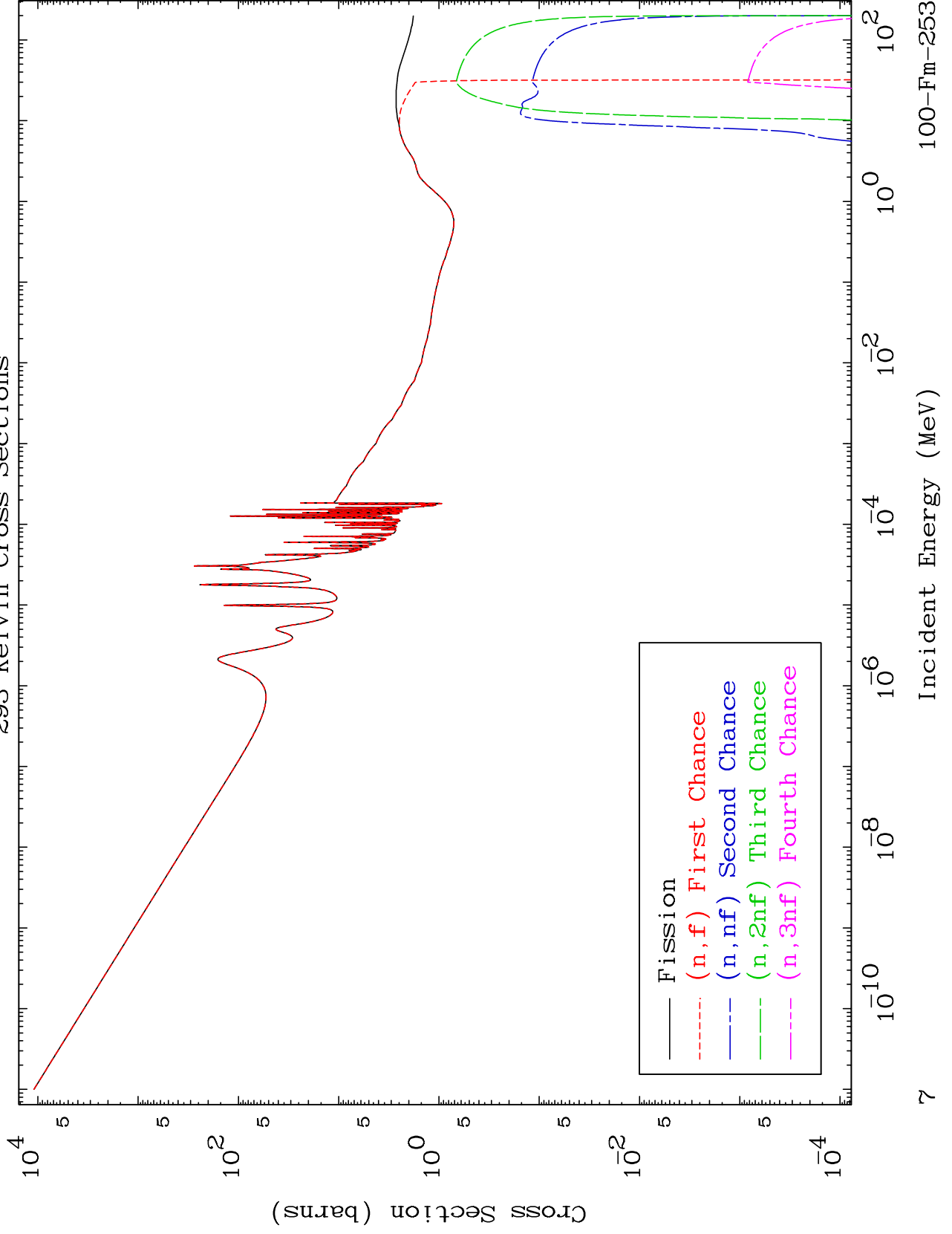
100-Fm-253



MAT 9934

Fission  
293 Kelvin Cross Sections

100-Fm-253



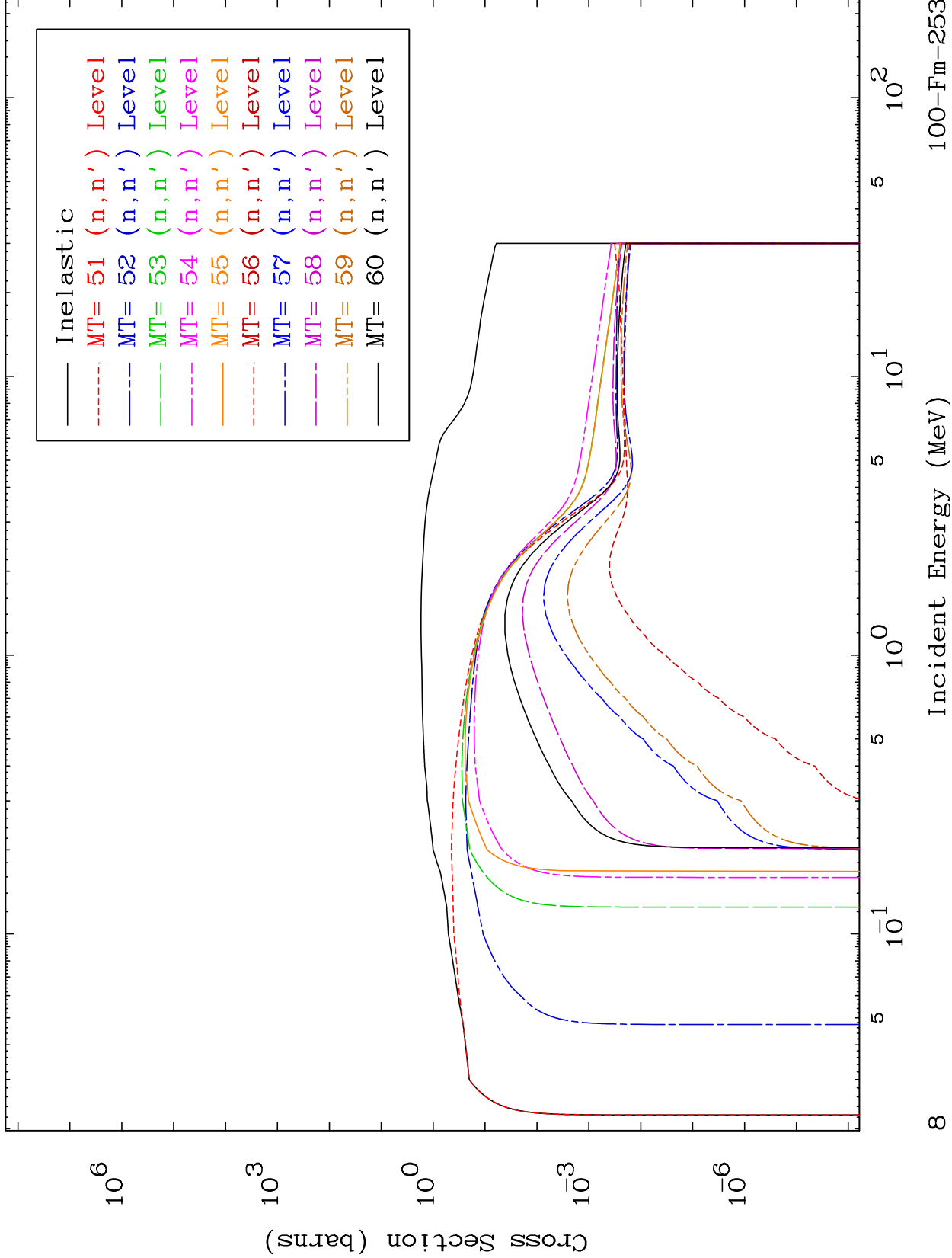


MAT 9934

(n,n') Level

100-Fm-253

293 Kelvin Cross Sections

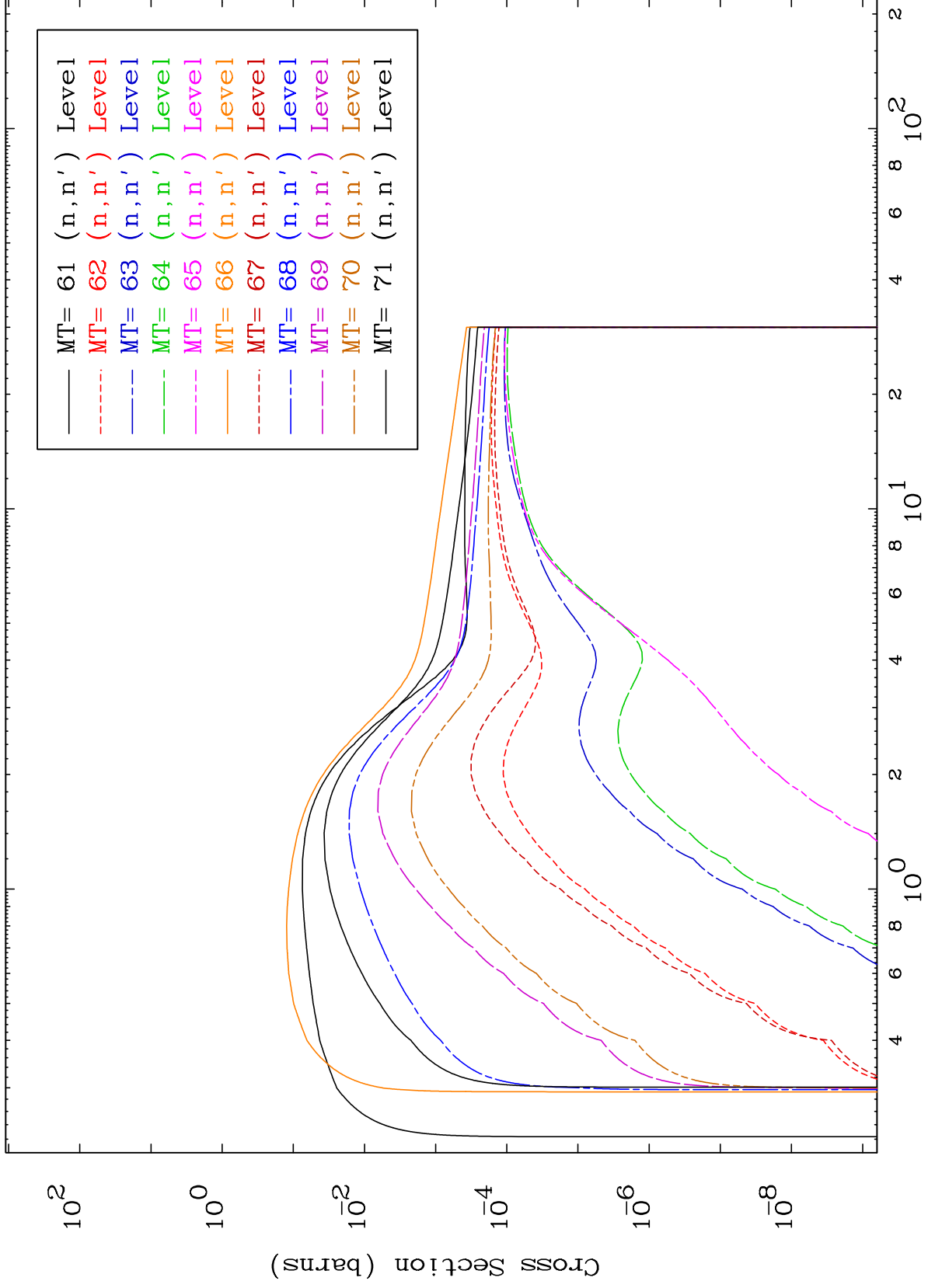


MAT 9934

(n,n') Level

100-Fm-253

293 Kelvin Cross Sections

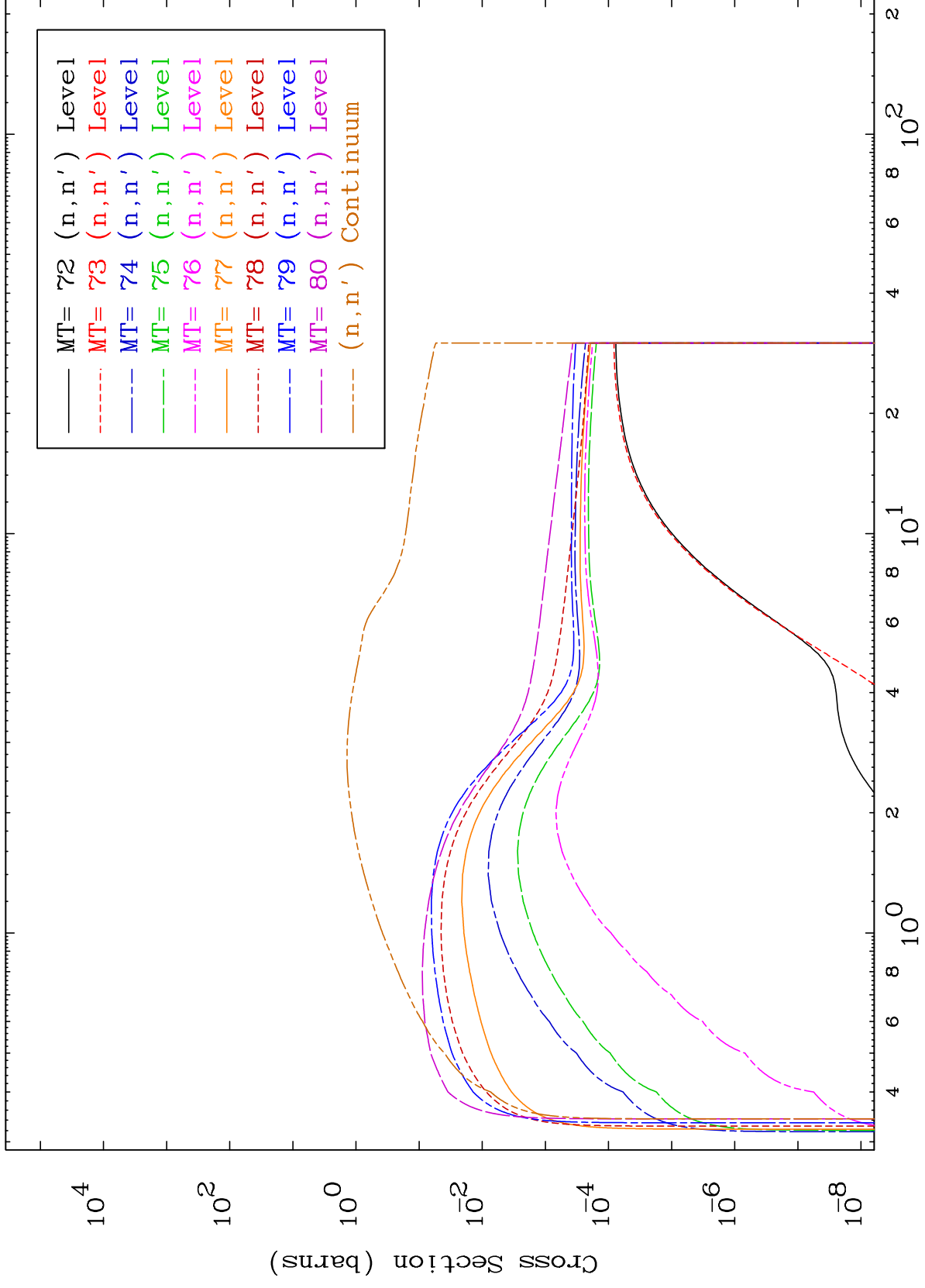


MAT 9934

(n,n') Level

100-Fm-253

293 Kelvin Cross Sections



10

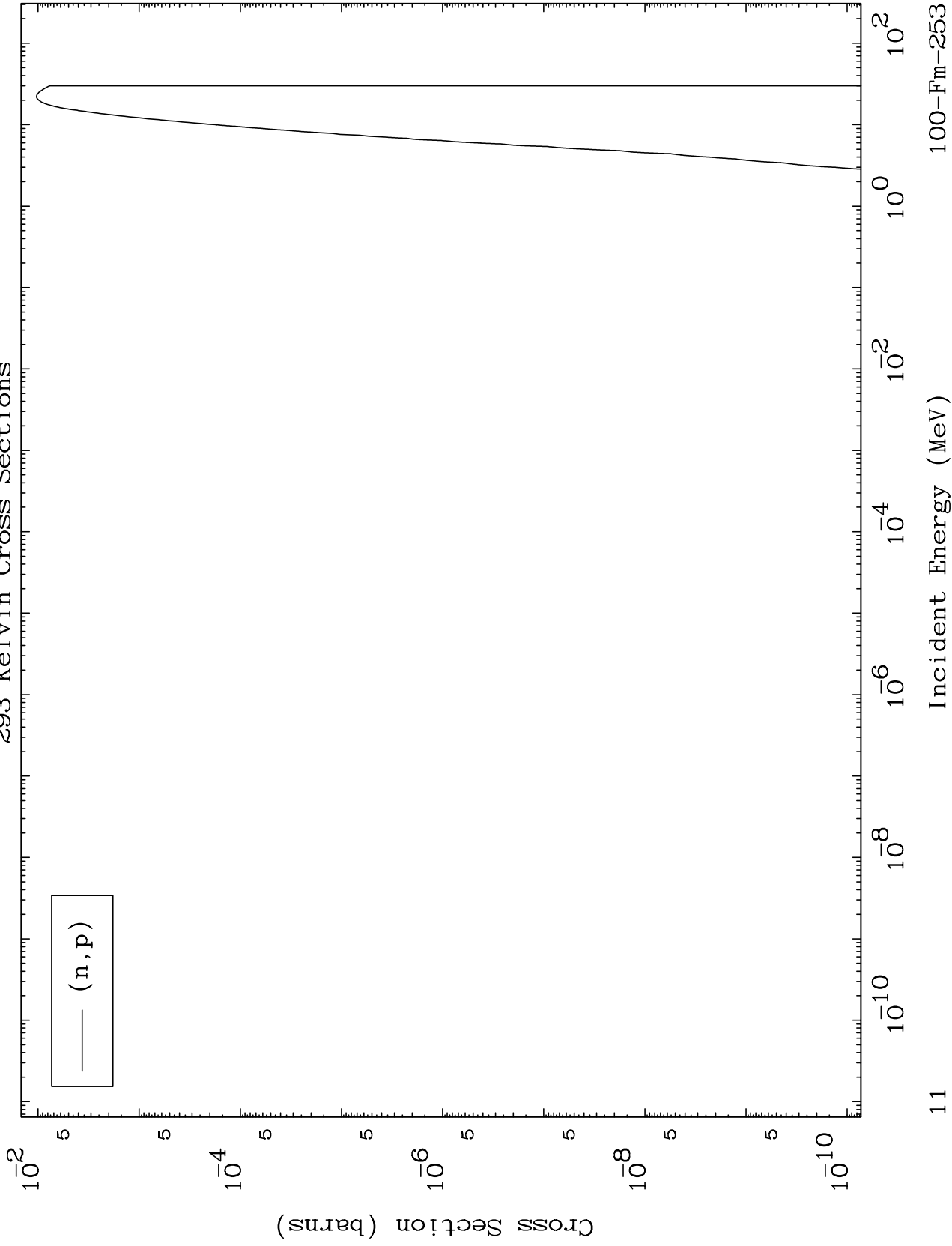
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,p) Levels  
293 Kelvin Cross Sections

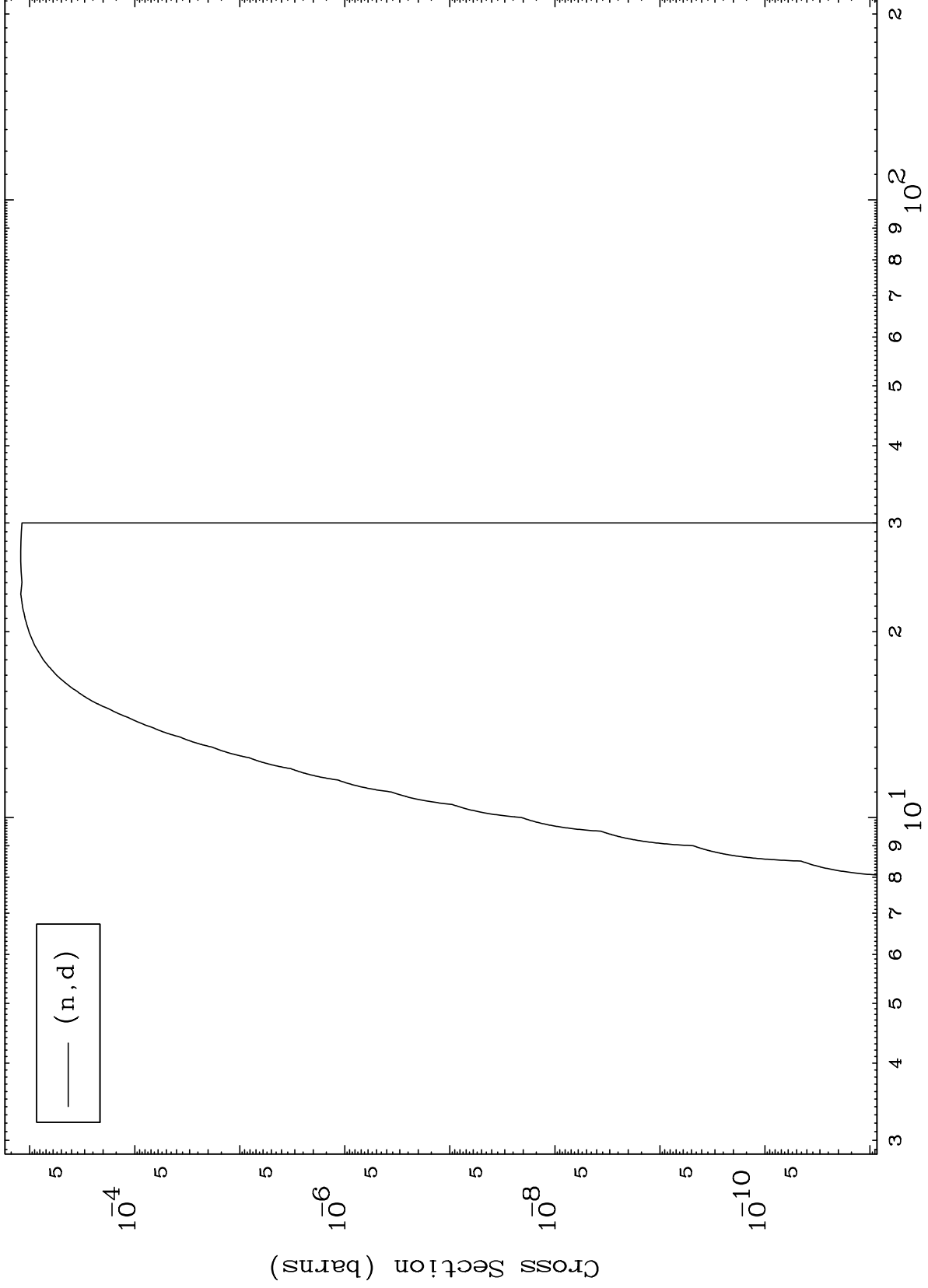
100-Fm-253



MAT 9934

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-253



(n,d)

12

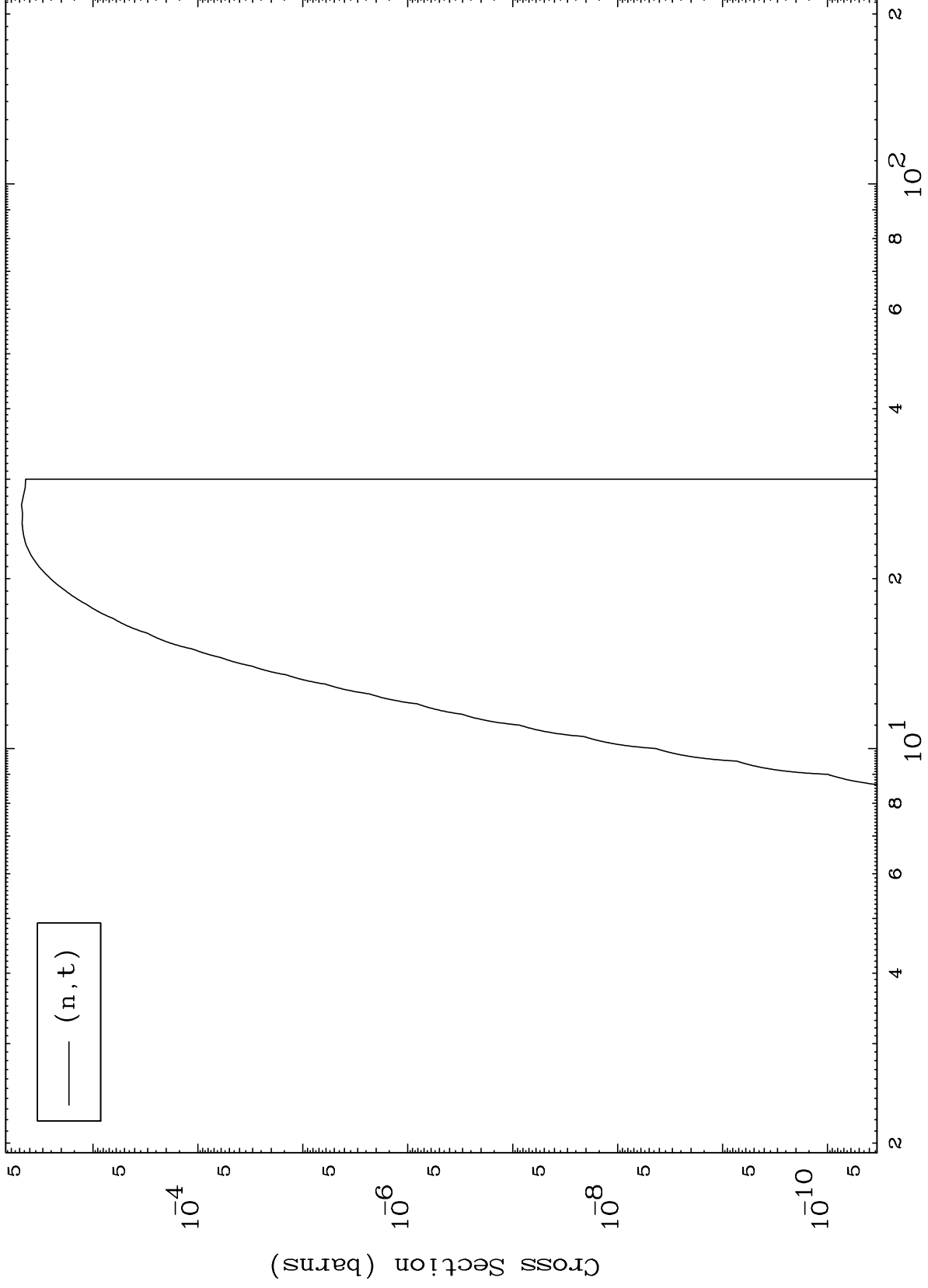
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-253



13

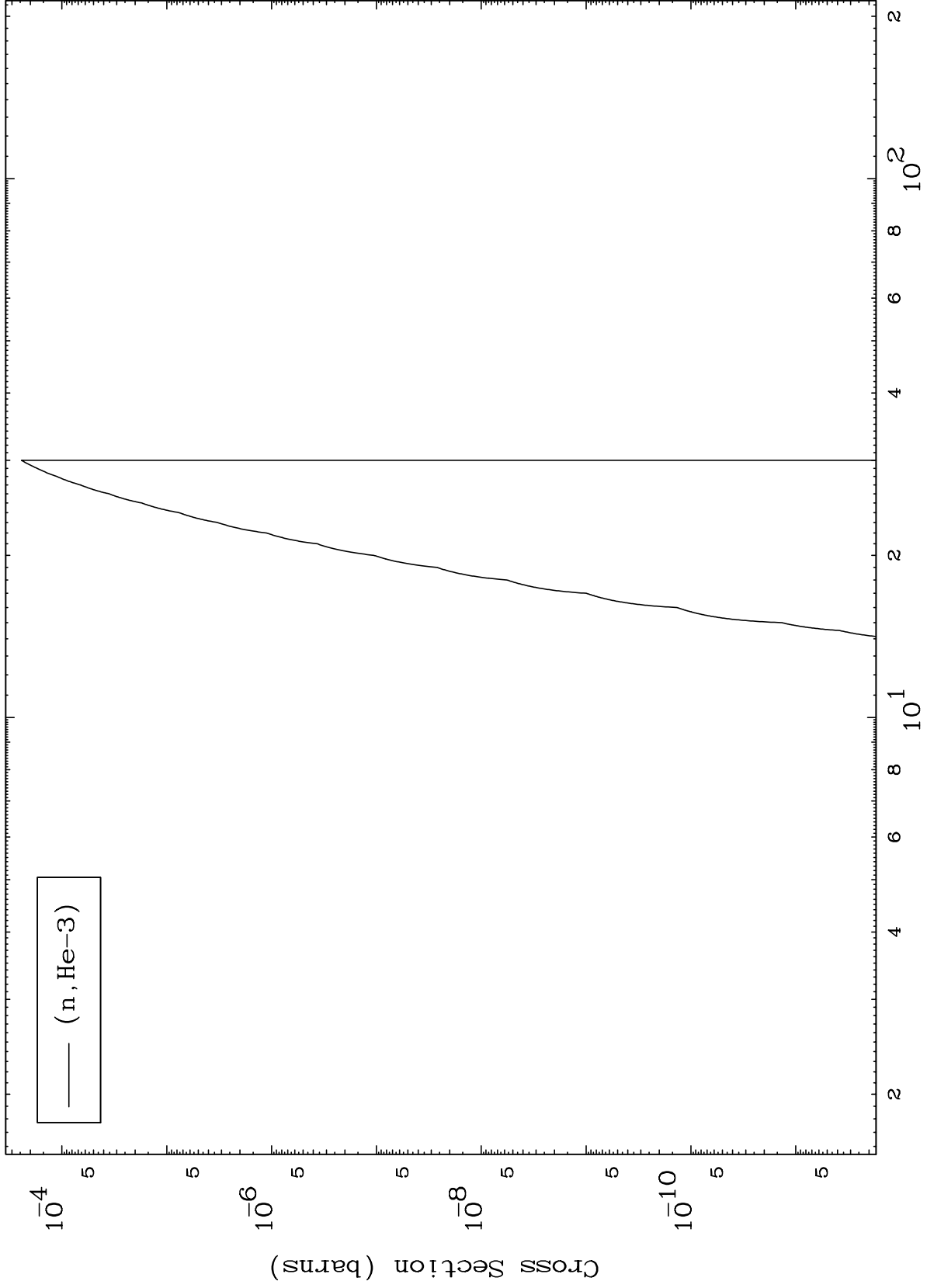
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,He3) Levels  
293 Kelvin Cross Sections

100-Fm-253



14

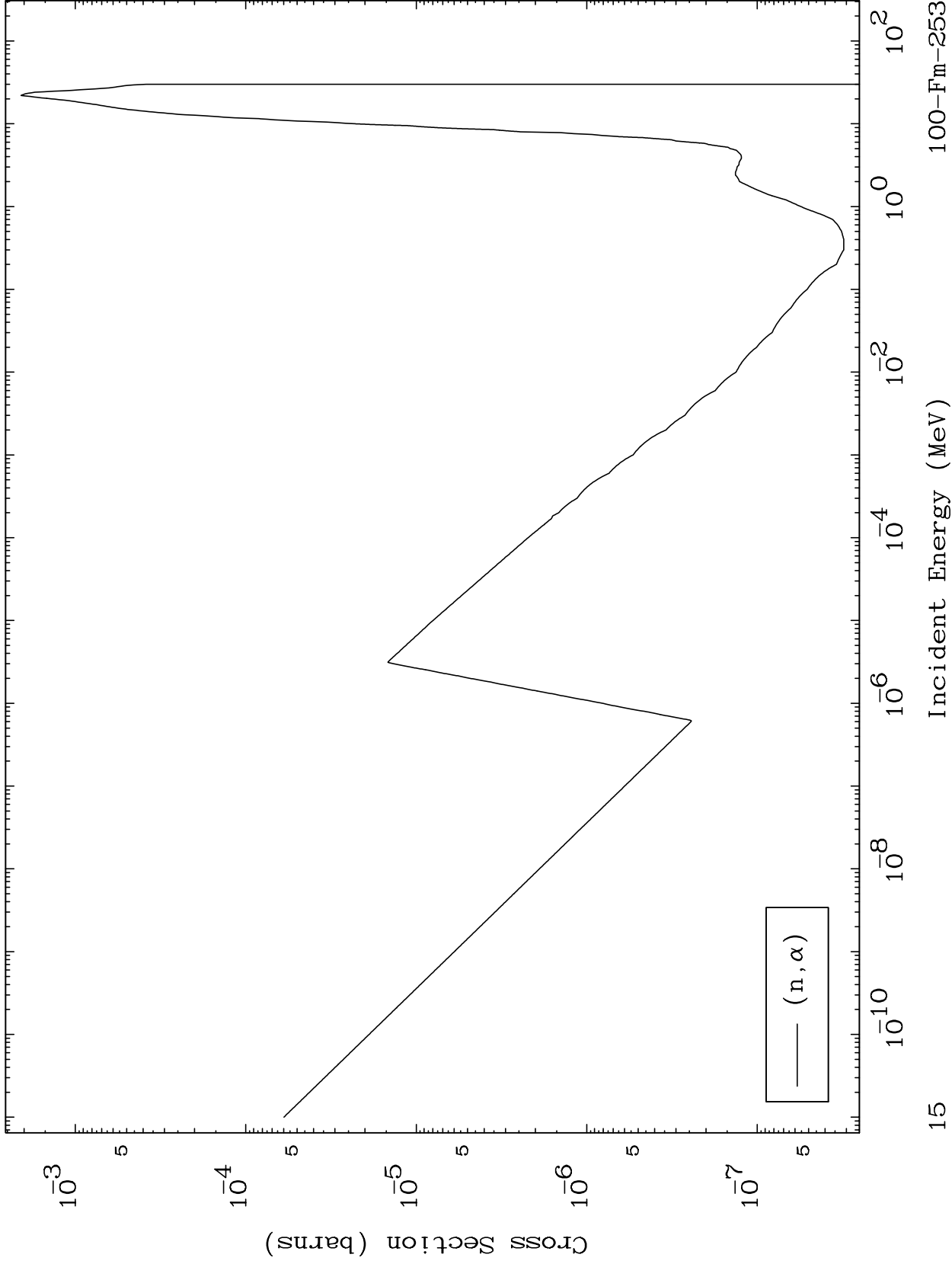
Incident Energy (MeV)

100-Fm-253

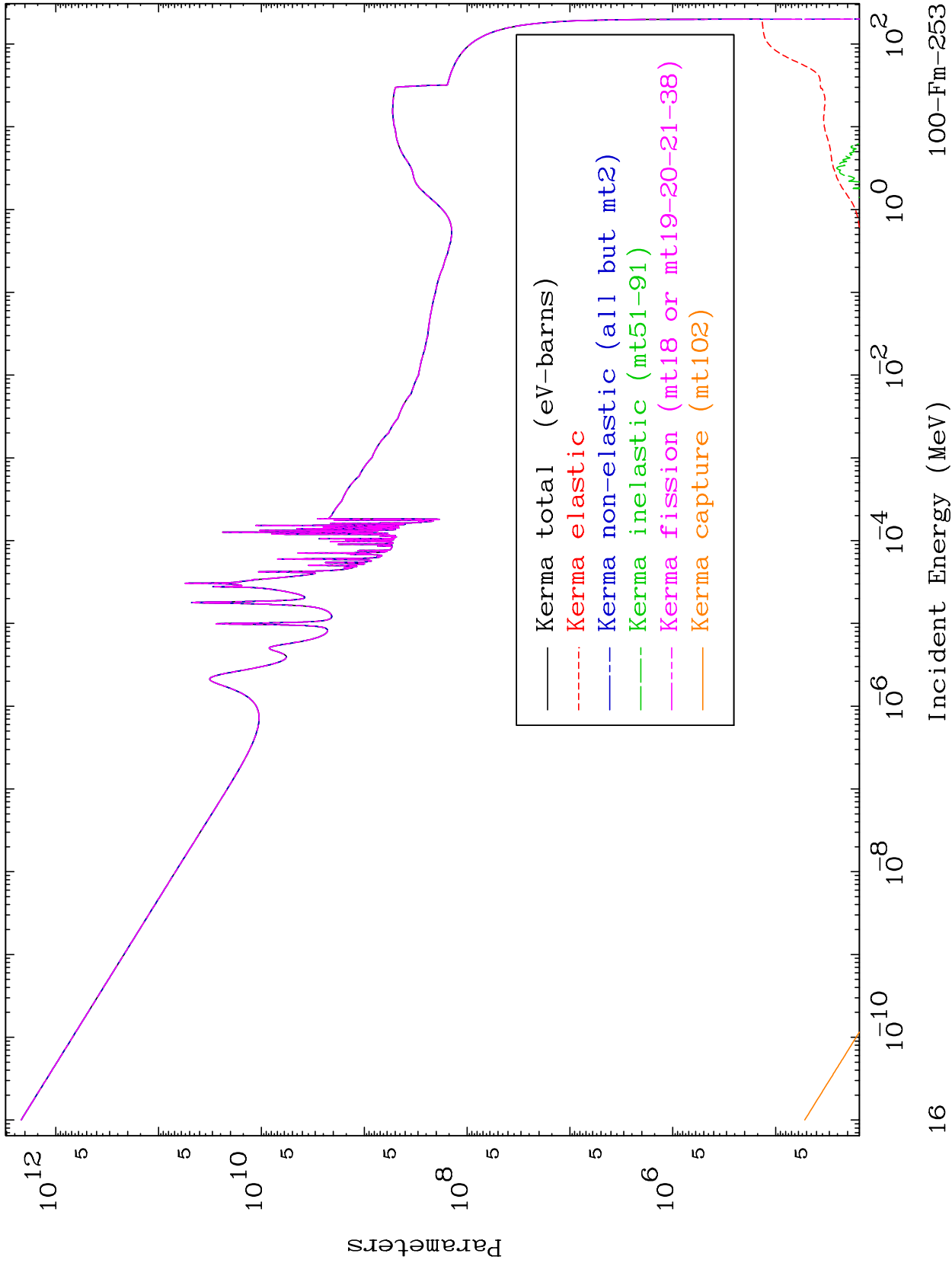
MAT 9934

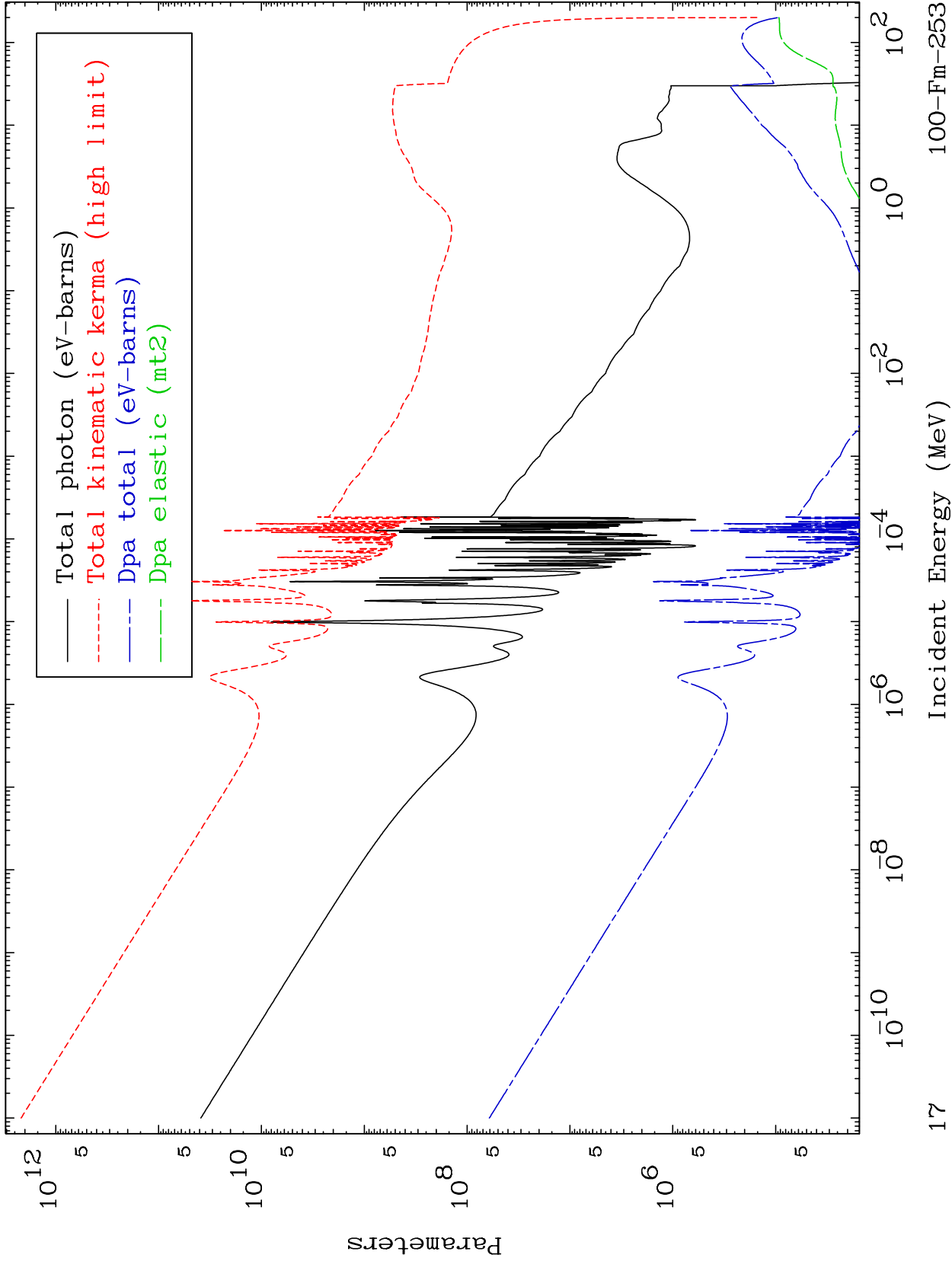
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

100-Fm-253





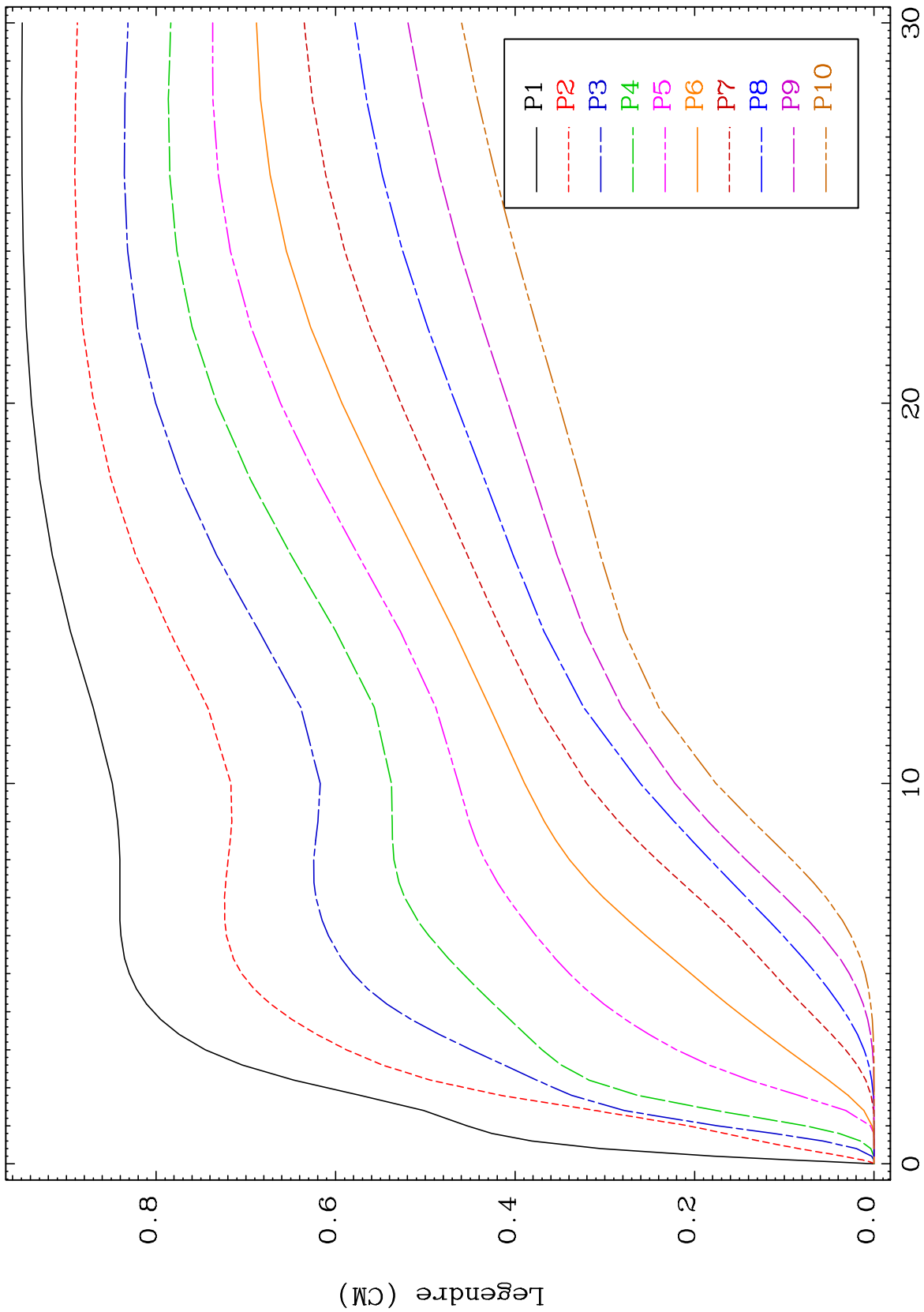




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Elastic Legendre Coefficients

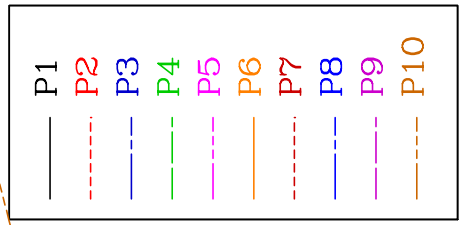
100-Fm-253

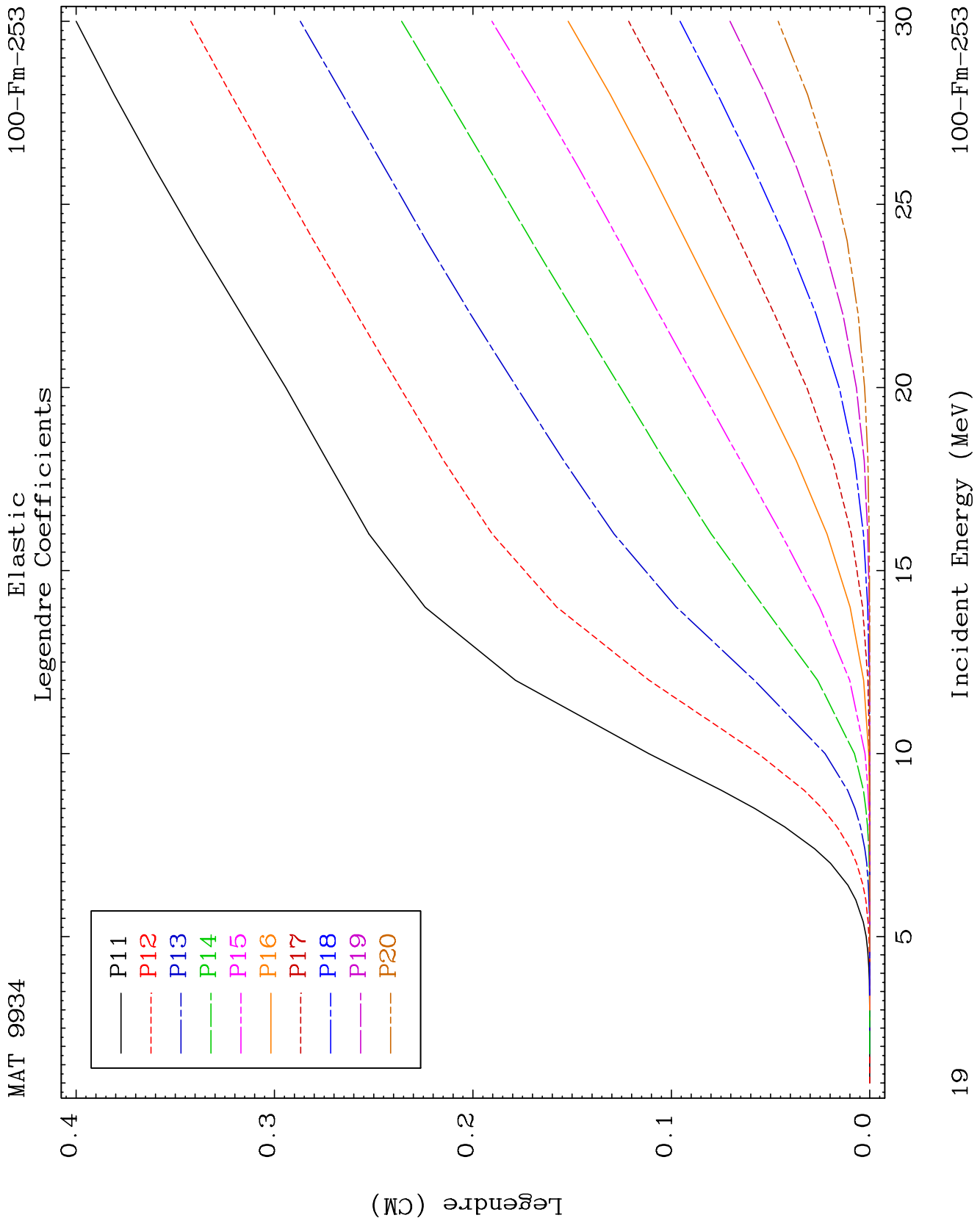


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Incident Energy (MeV)

100-Fm-253

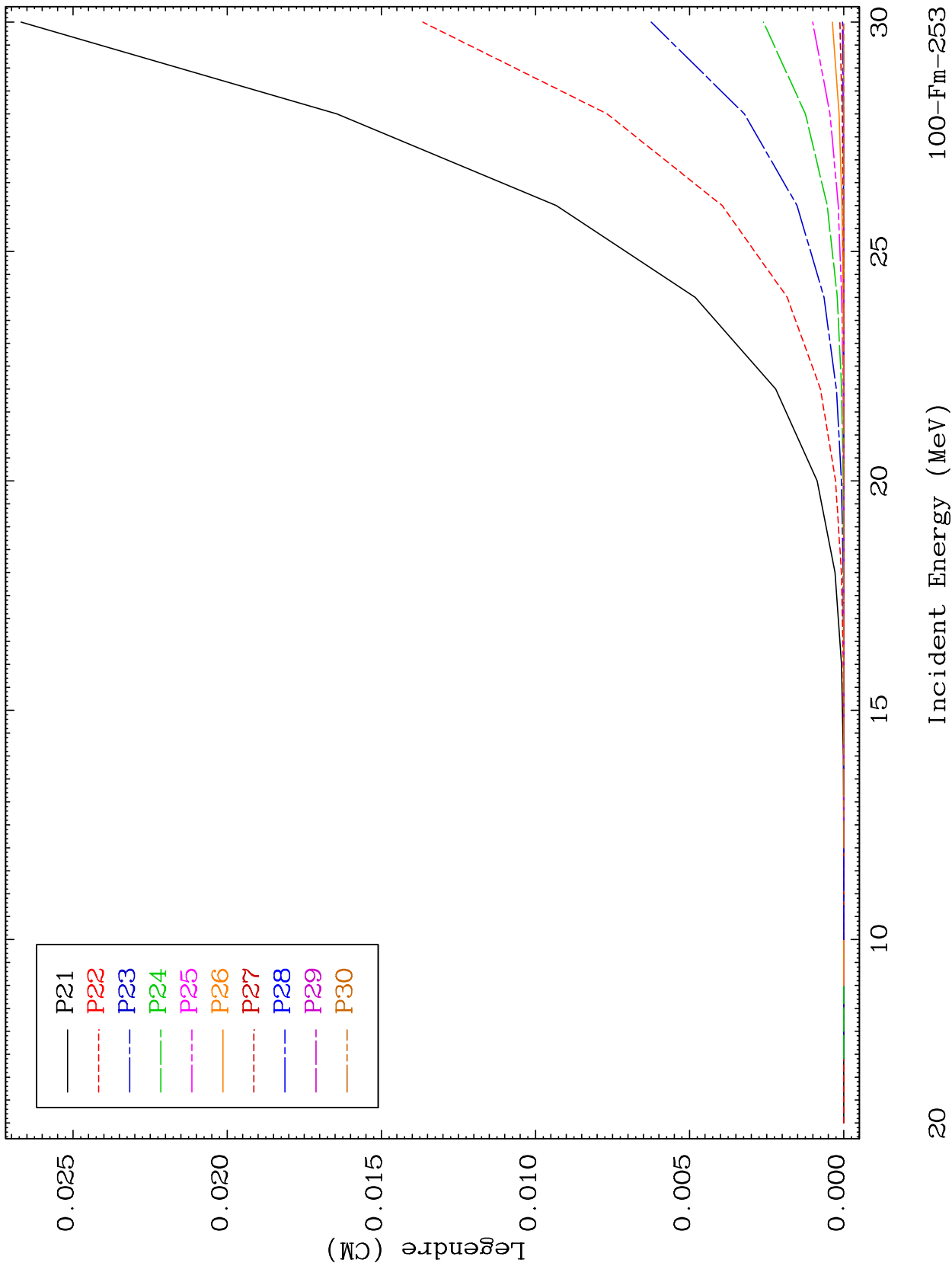




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Elastic Legendre Coefficients

100-Fm-253



20

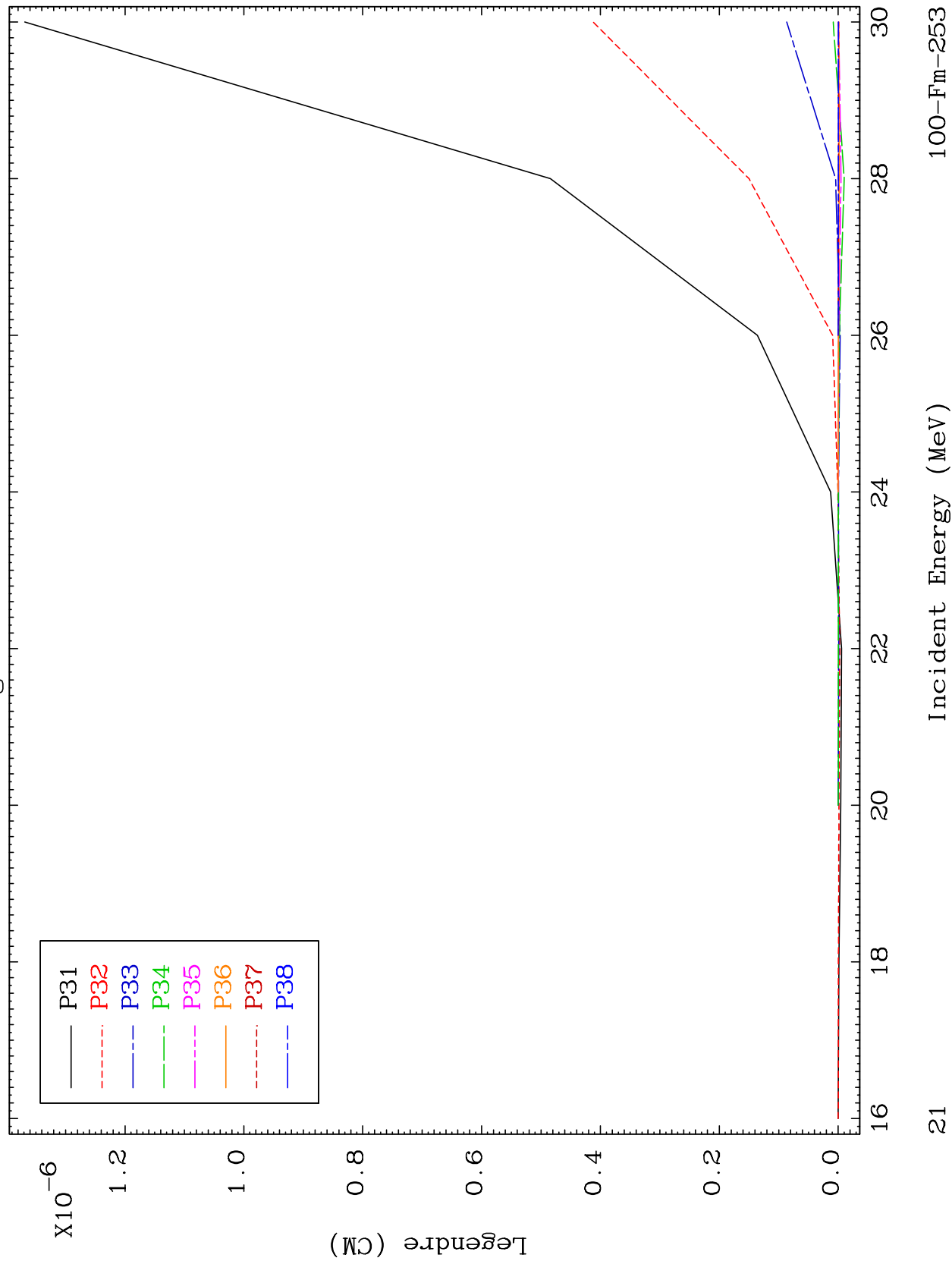
Incident Energy (MeV)

100-Fm-253

MAT 9934

Elastic  
Legendre Coefficients

100-Fm-253



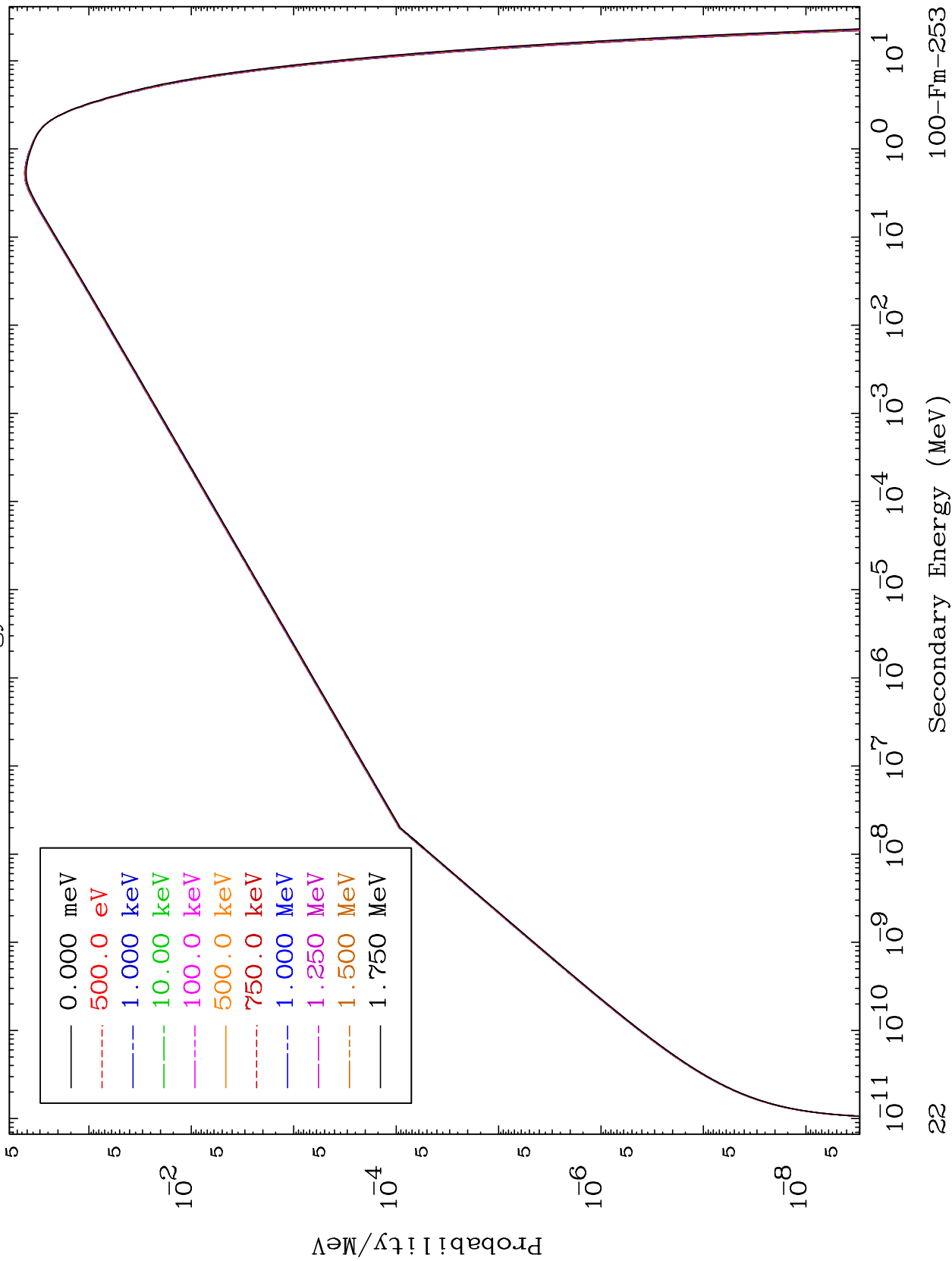
21

100-Fm-253

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### Fission Energy Distribution

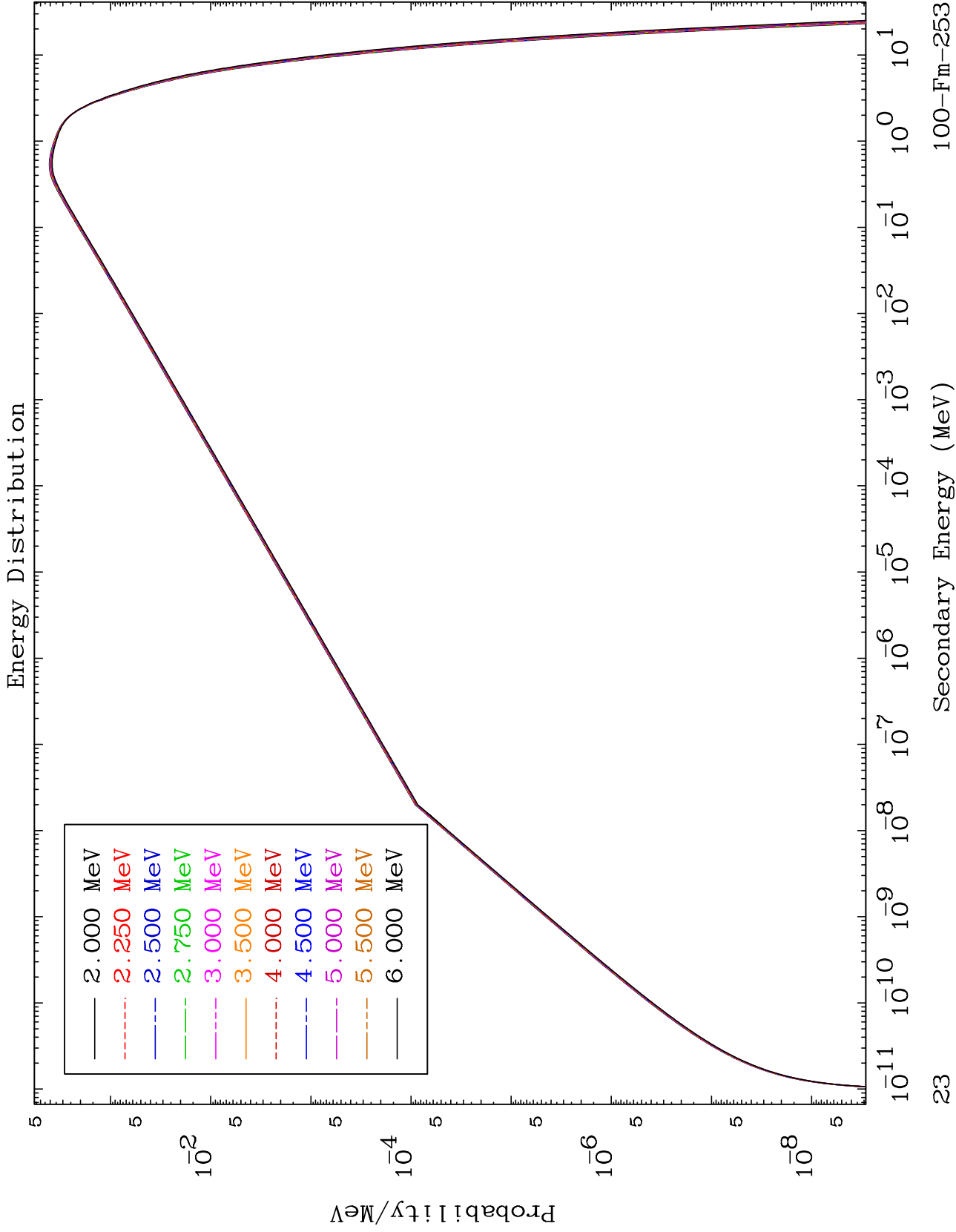
100-Fm-253



MAT 9934

Fission

100-Fm-253

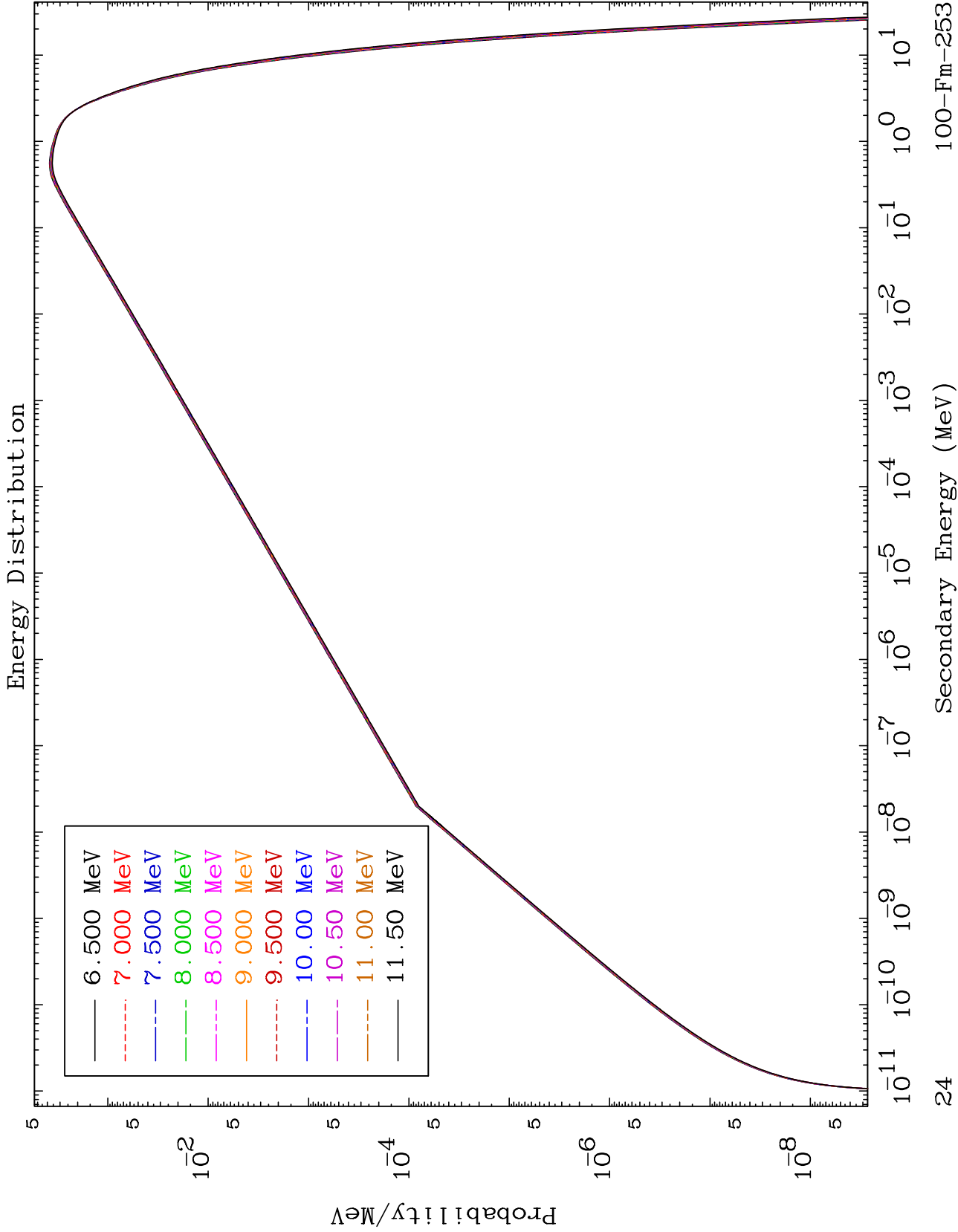




MAT 9934

Fission

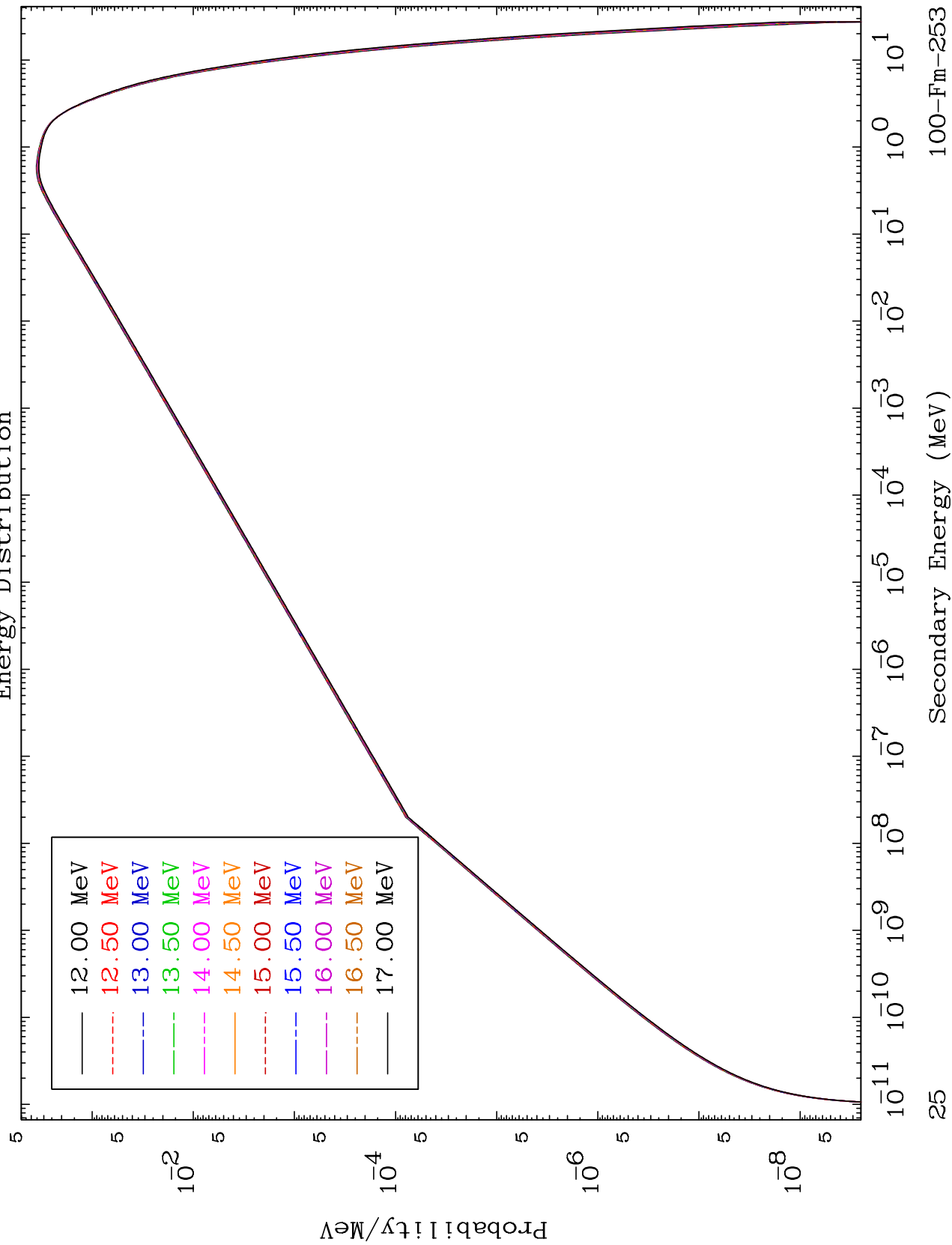
100-Fm-253



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### Fission Energy Distribution

100-Fm-253



100-Fm-253

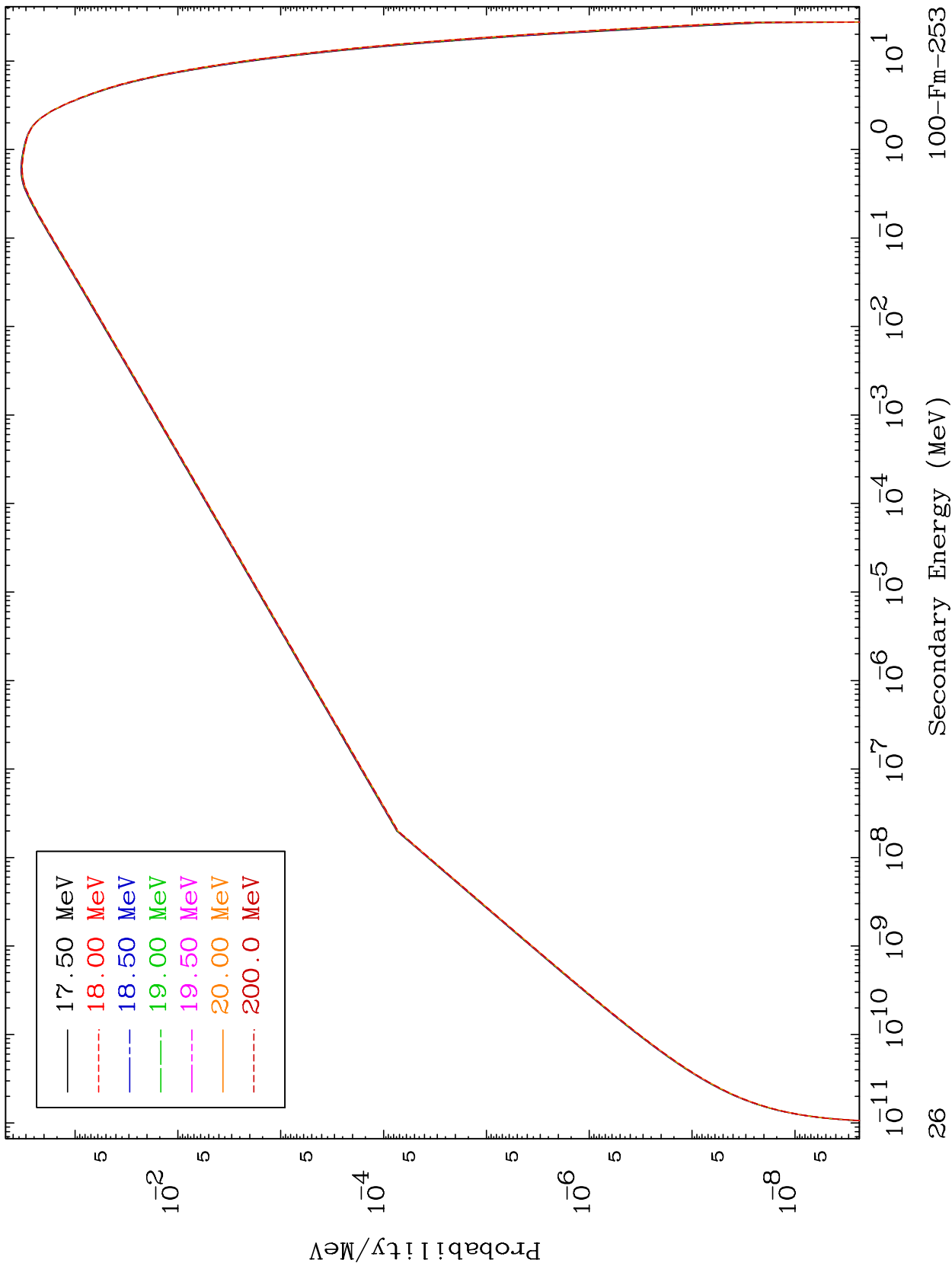
Secondary Energy (MeV)

25

MAT 9934

Fission Energy Distribution

100-Fm-253

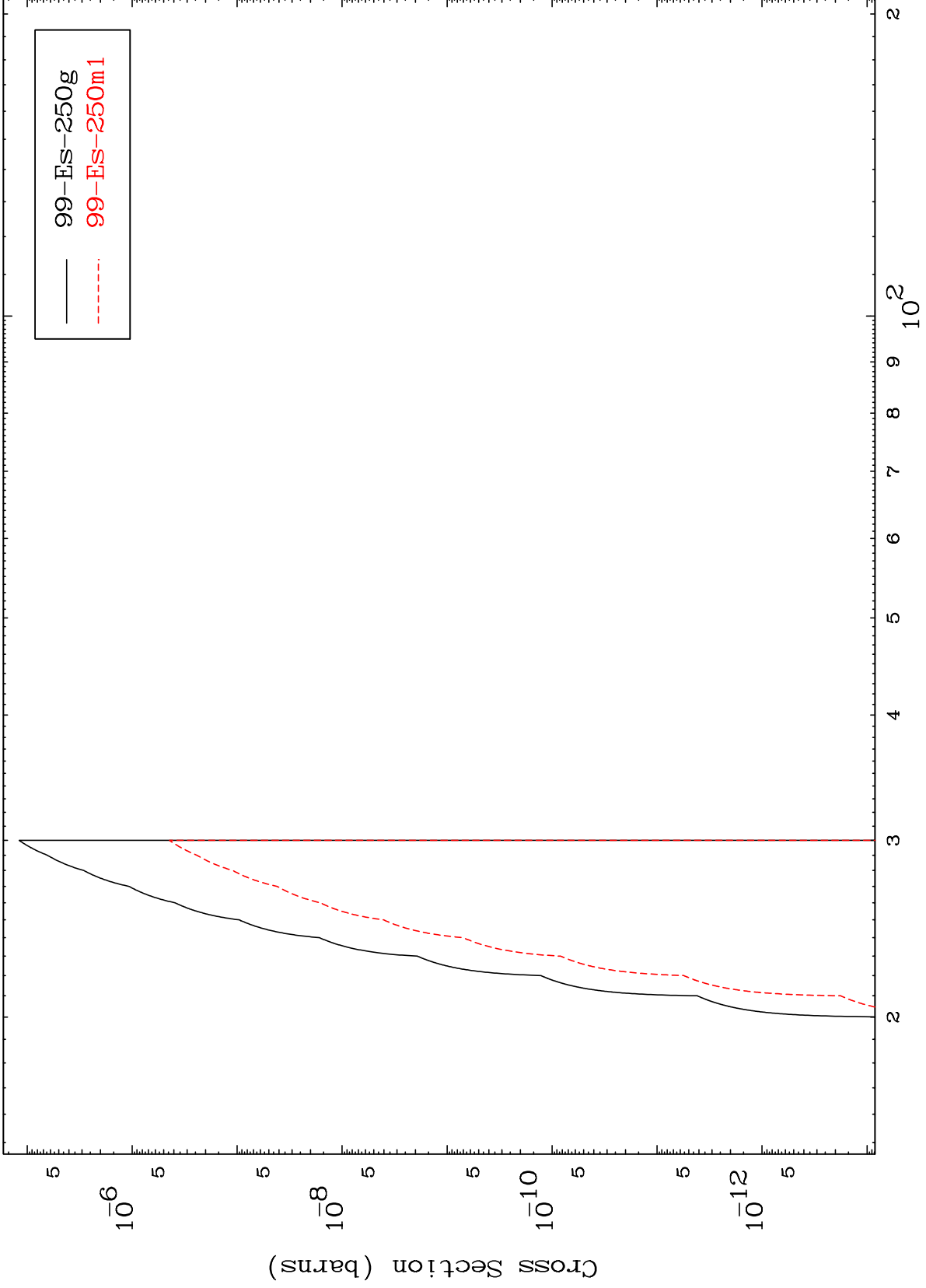


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(n,2n) d

100-Fm-253

Radionuclide Production Cross Section



27

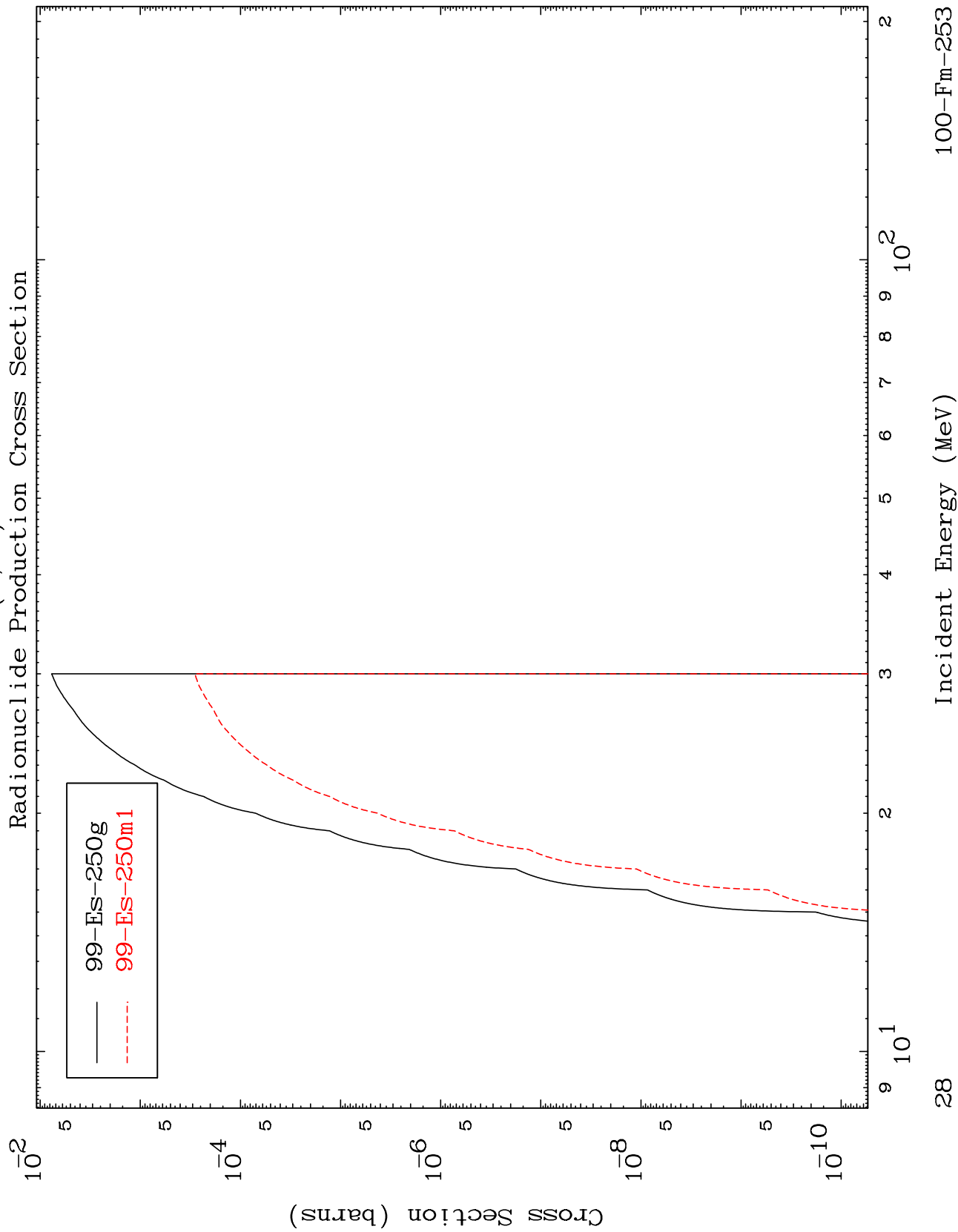
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,n') t

100-Fm-253



28

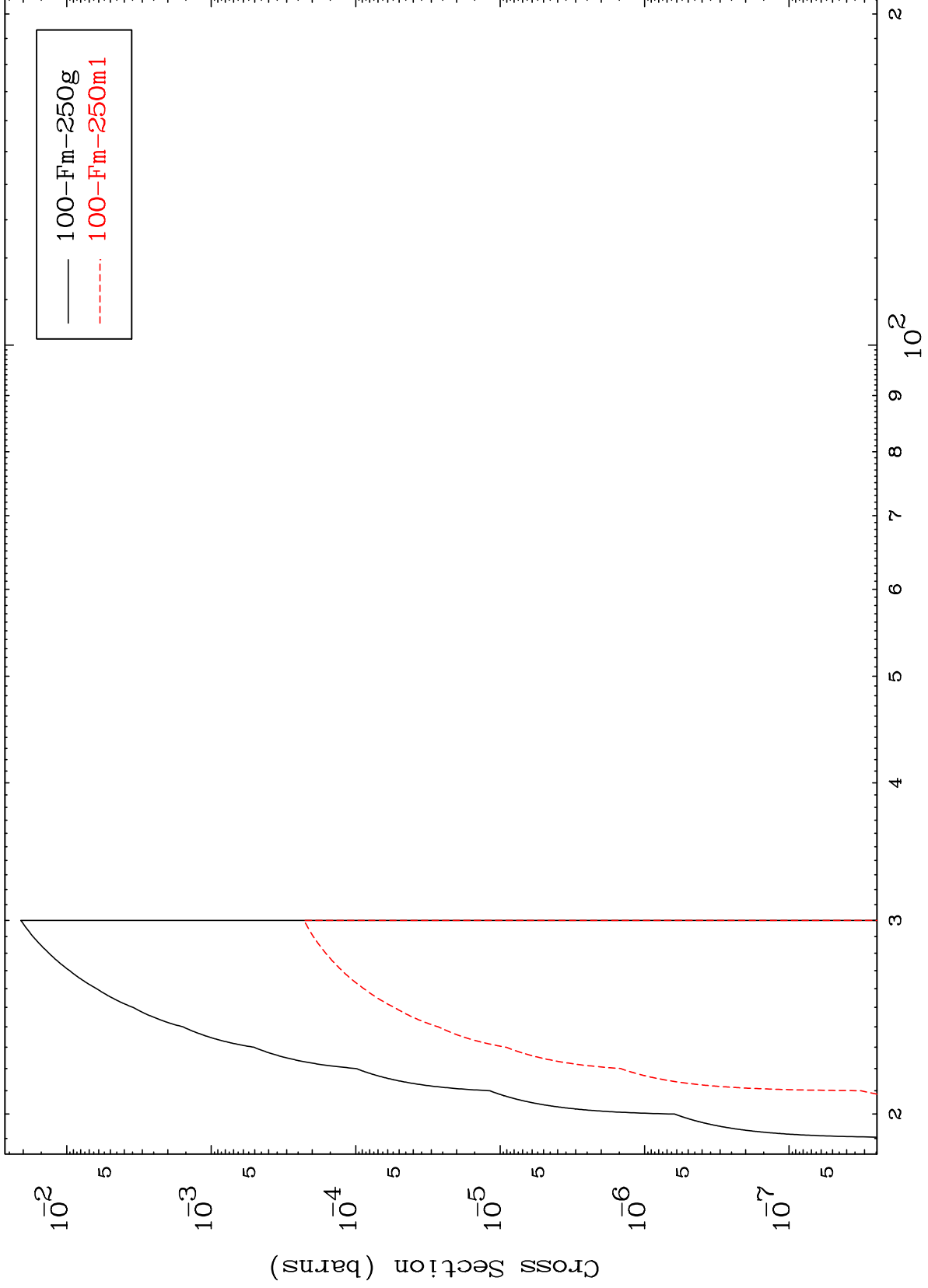
100-Fm-253

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(n,4n)

100-Fm-253

Radionuclide Production Cross Section



29

Incident Energy (MeV)

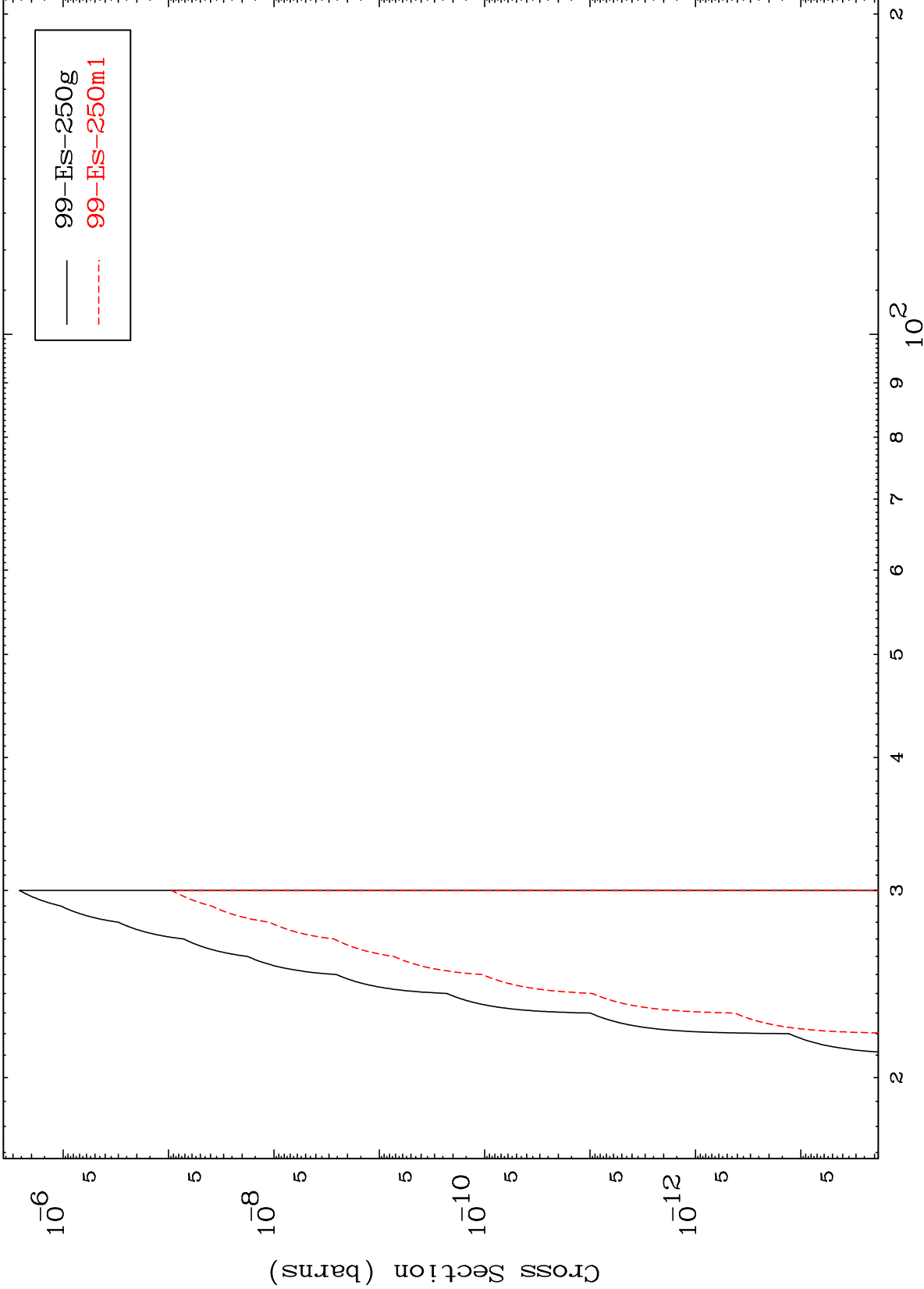
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(n,3n) p

100-Fm-253

Radionuclide Production Cross Section



30

Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,n') p  $\alpha$

100-Fm-253

Radionuclide Production Cross Section

