

Program EVALPLOT
(Version 2018-1)

by

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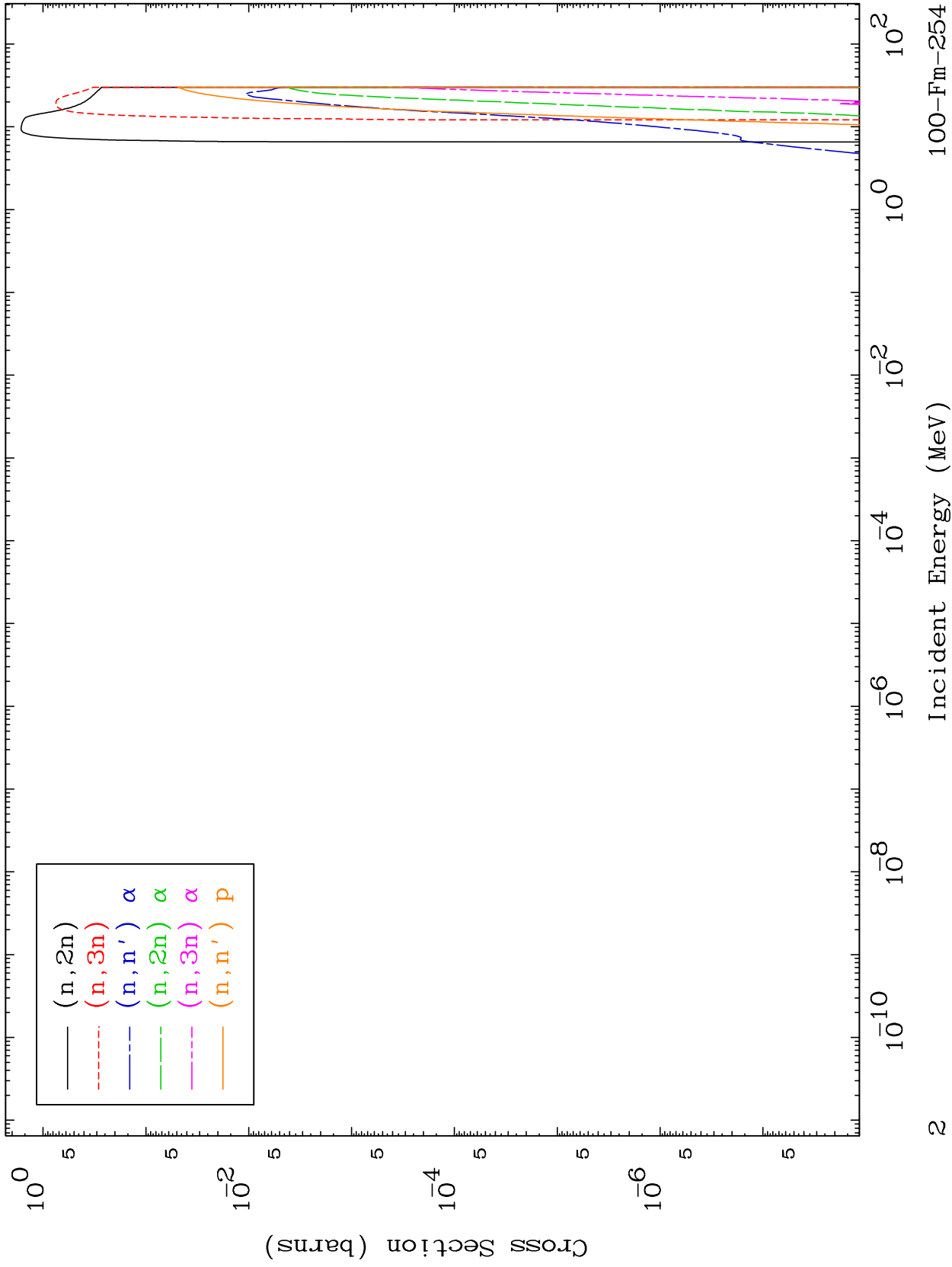
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9935

Neutron Production
293 Kelvin Cross Sections

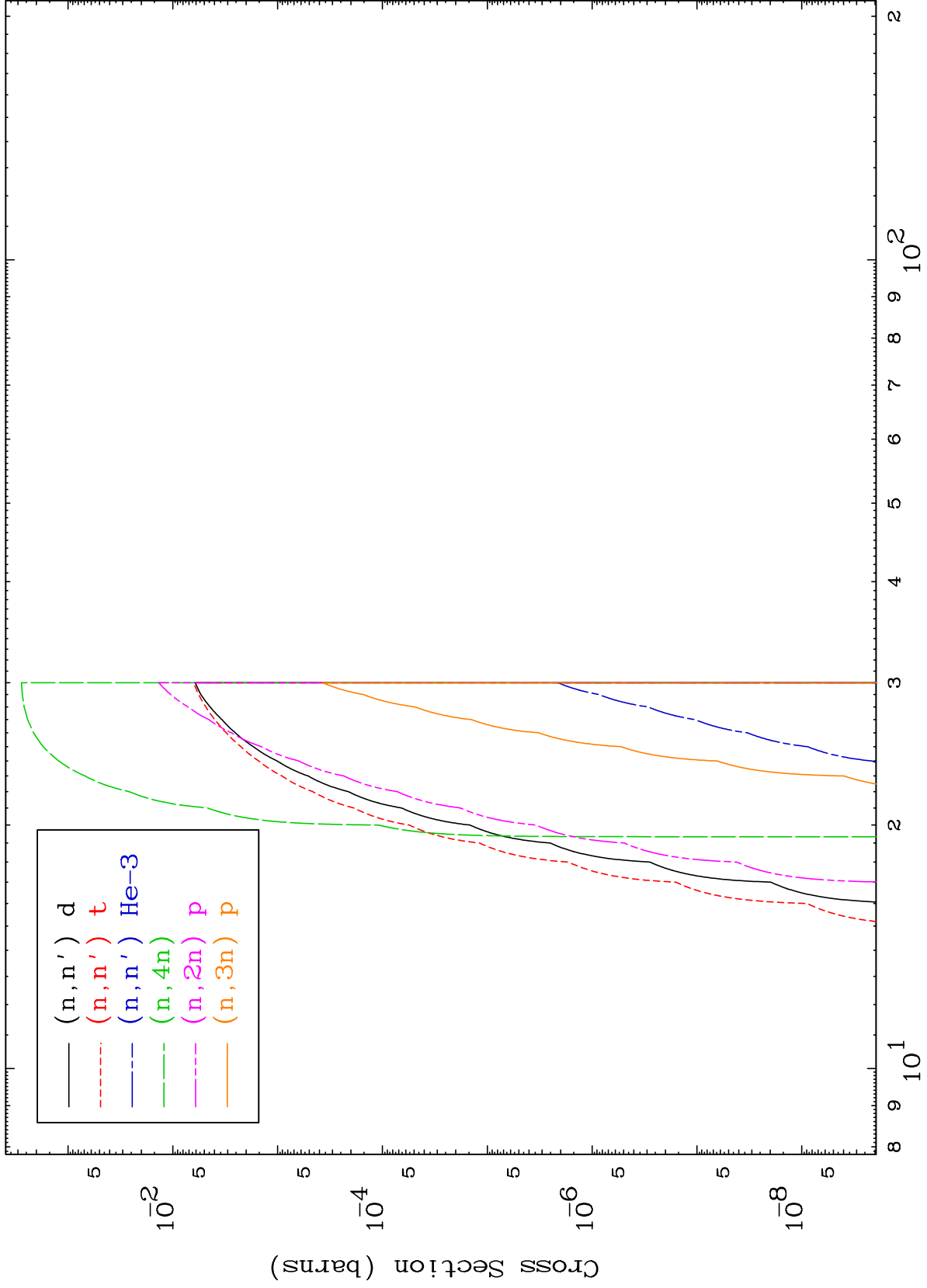
100-Fm-254



MAT 9935

Neutron Production
293 Kelvin Cross Sections

100-Fm-254



Incident Energy (MeV)

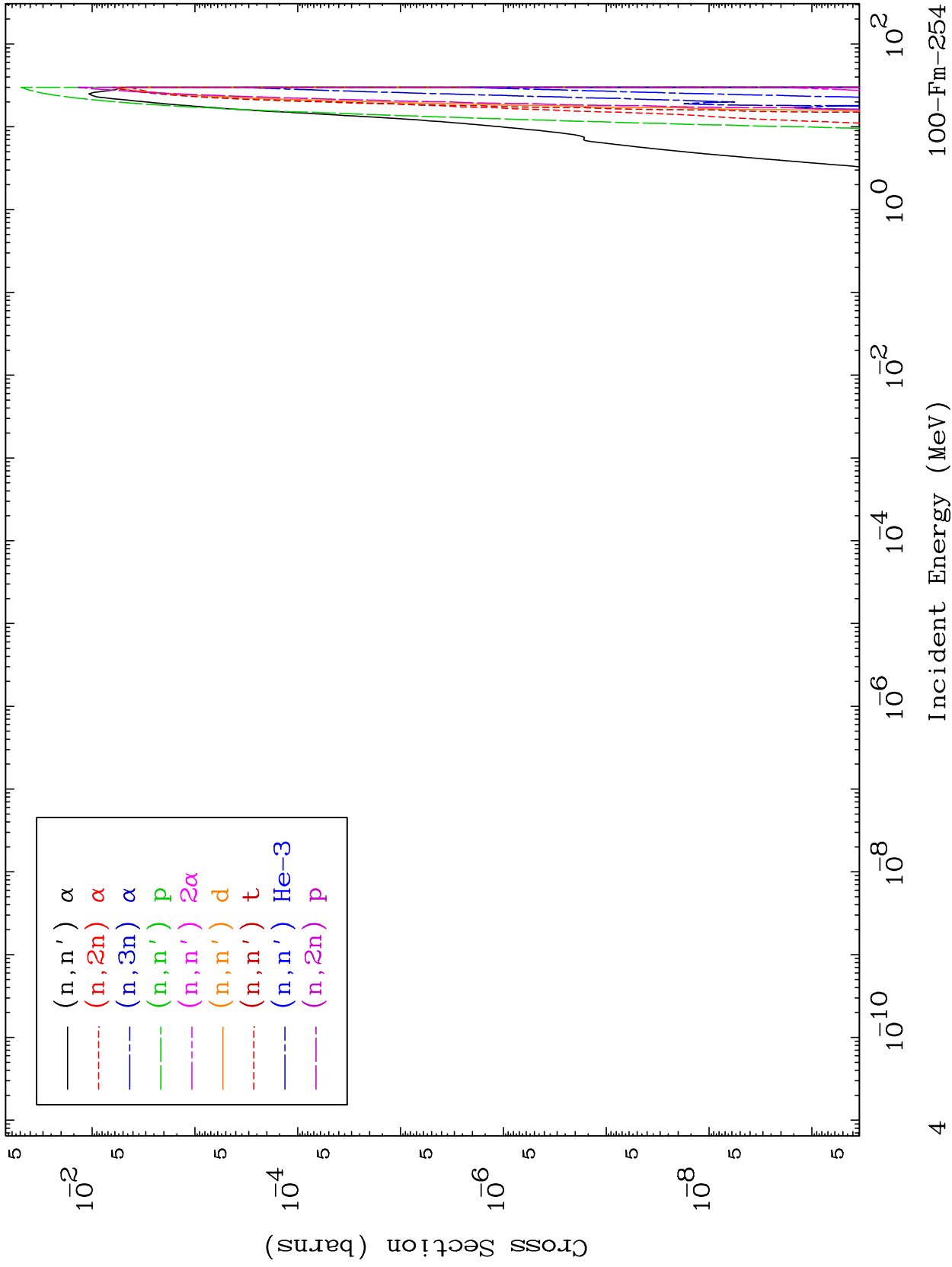
100-Fm-254

3

MAT 9935

Charged Particle
293 Kelvin Cross Sections

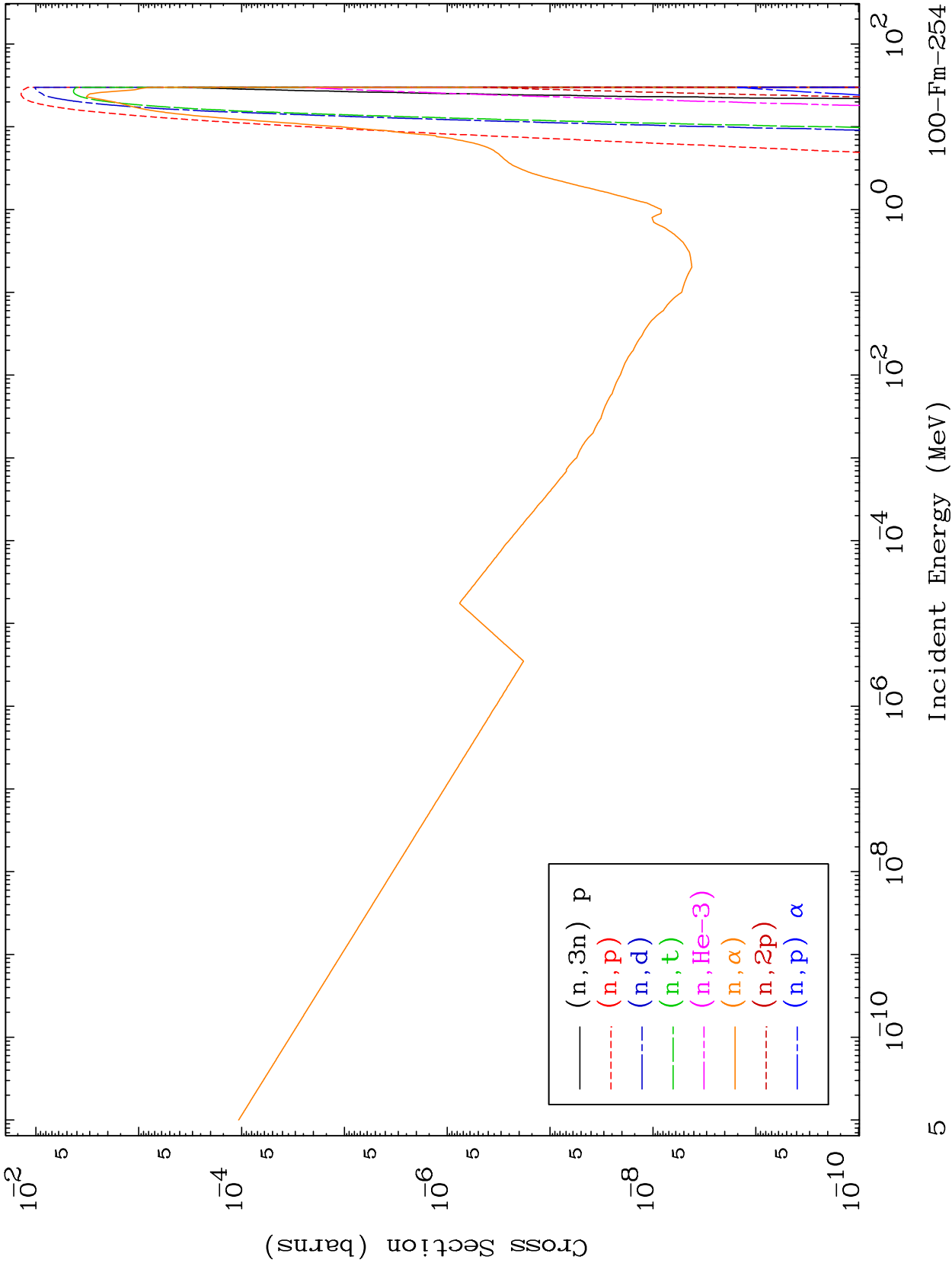
100-Fm-254



MAT 9935

Charged Particle
293 Kelvin Cross Sections

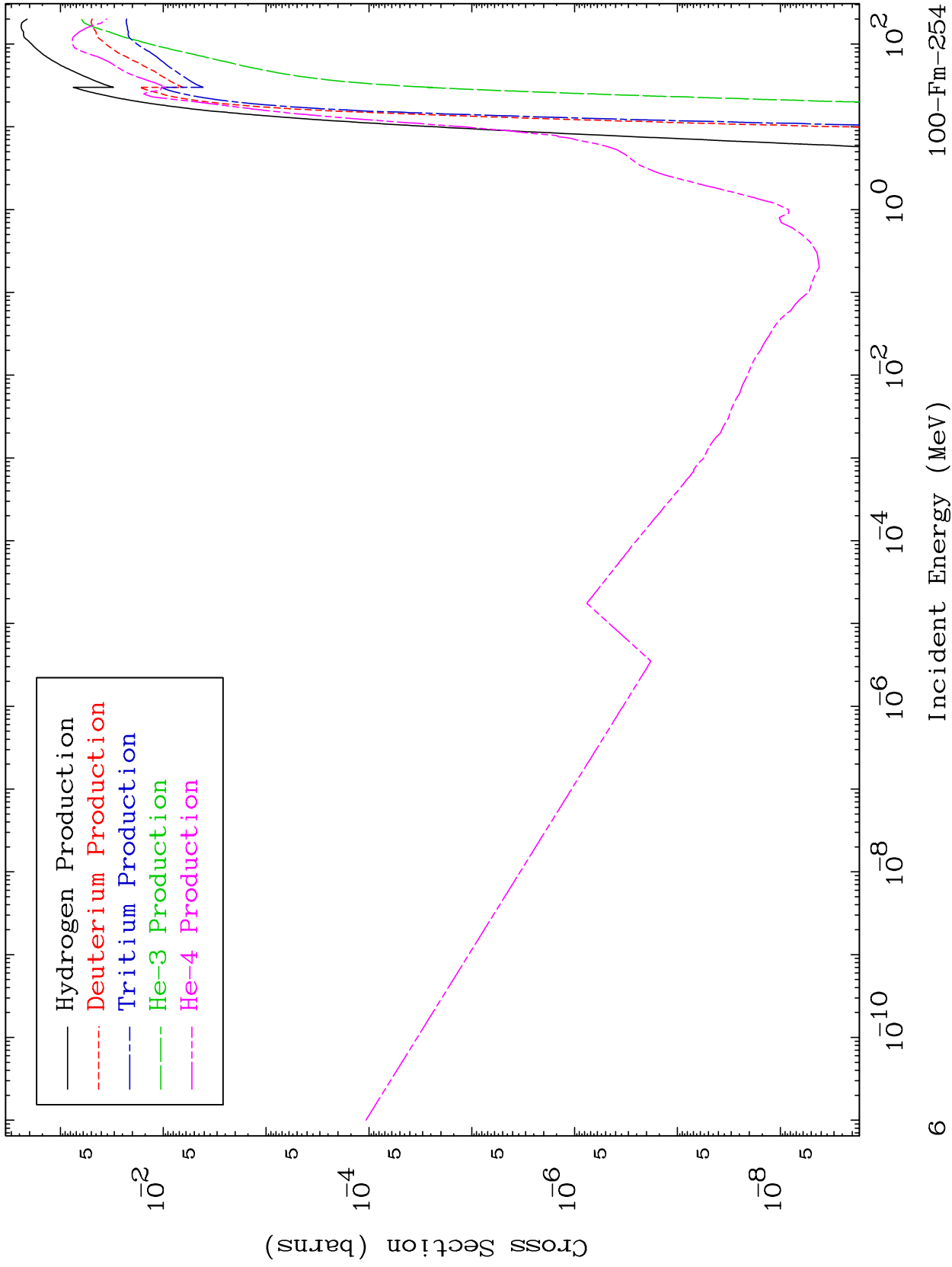
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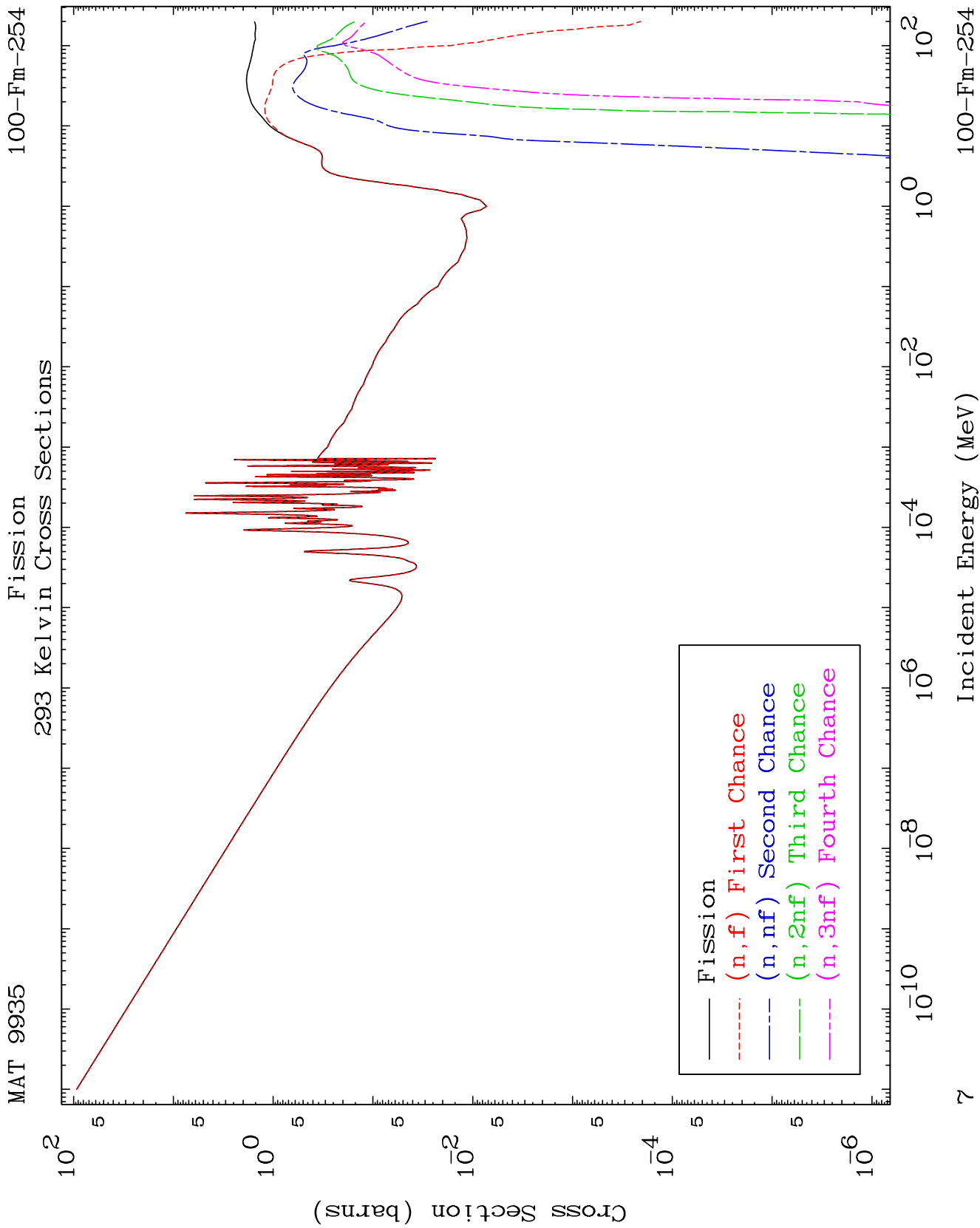


MAT 9935

Particle Production
293 Kelvin Cross Sections

100-Fm-254



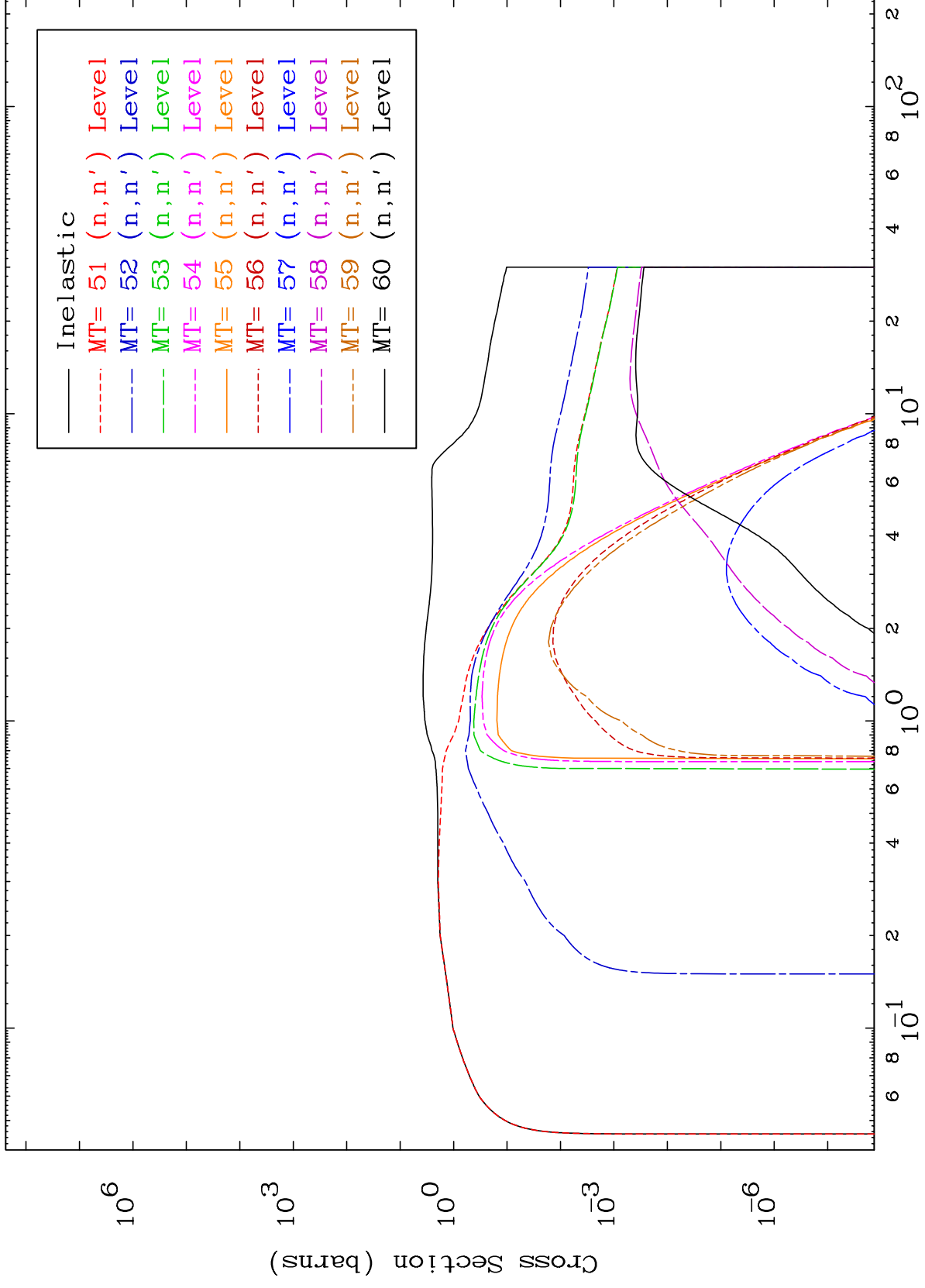


MAT 9935

(n,n') Level

100-Fm-254

293 Kelvin Cross Sections

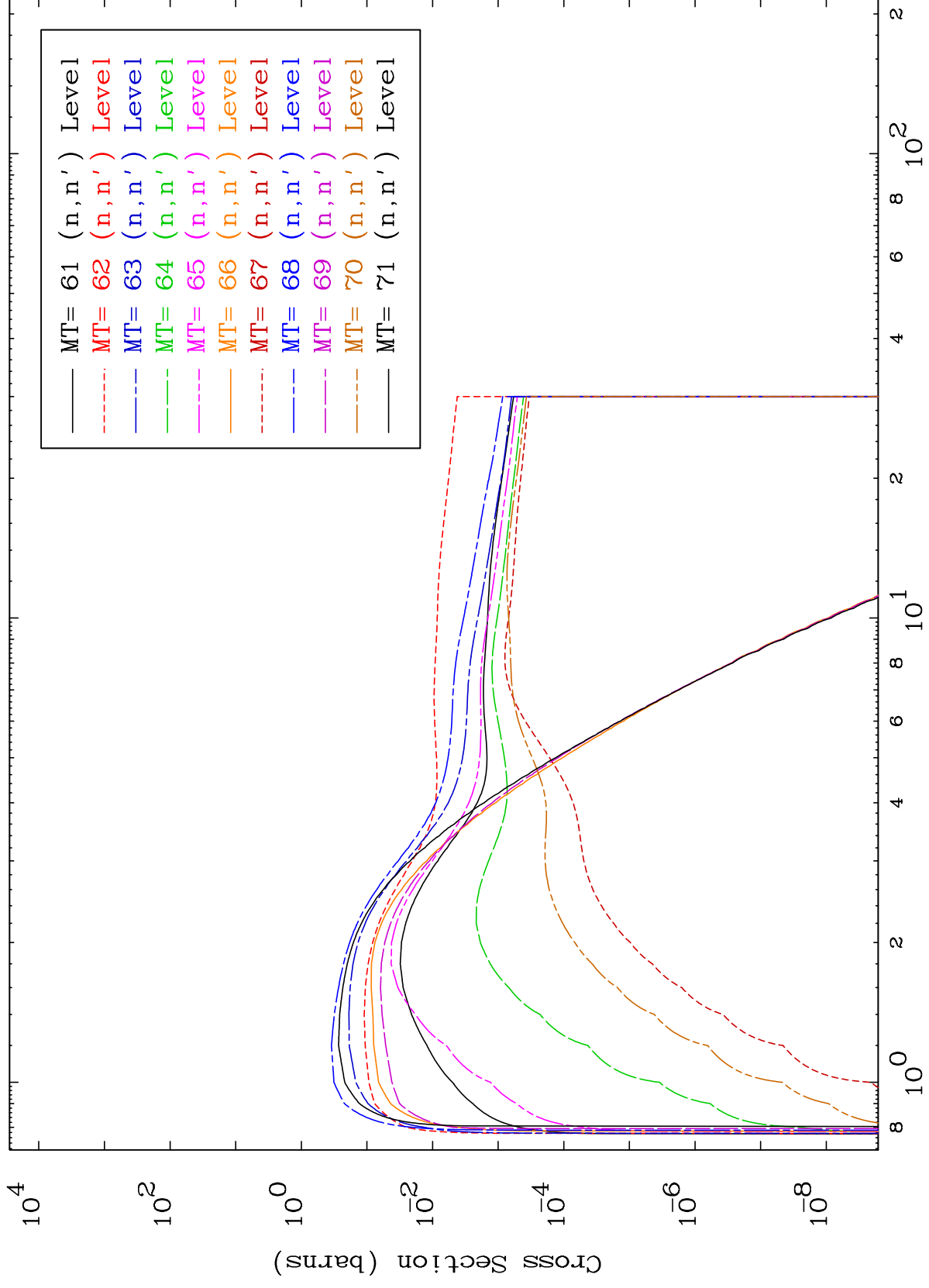


MAT 9935

(n,n') Level

100-Fm-254

293 Kelvin Cross Sections



9

Incident Energy (MeV)

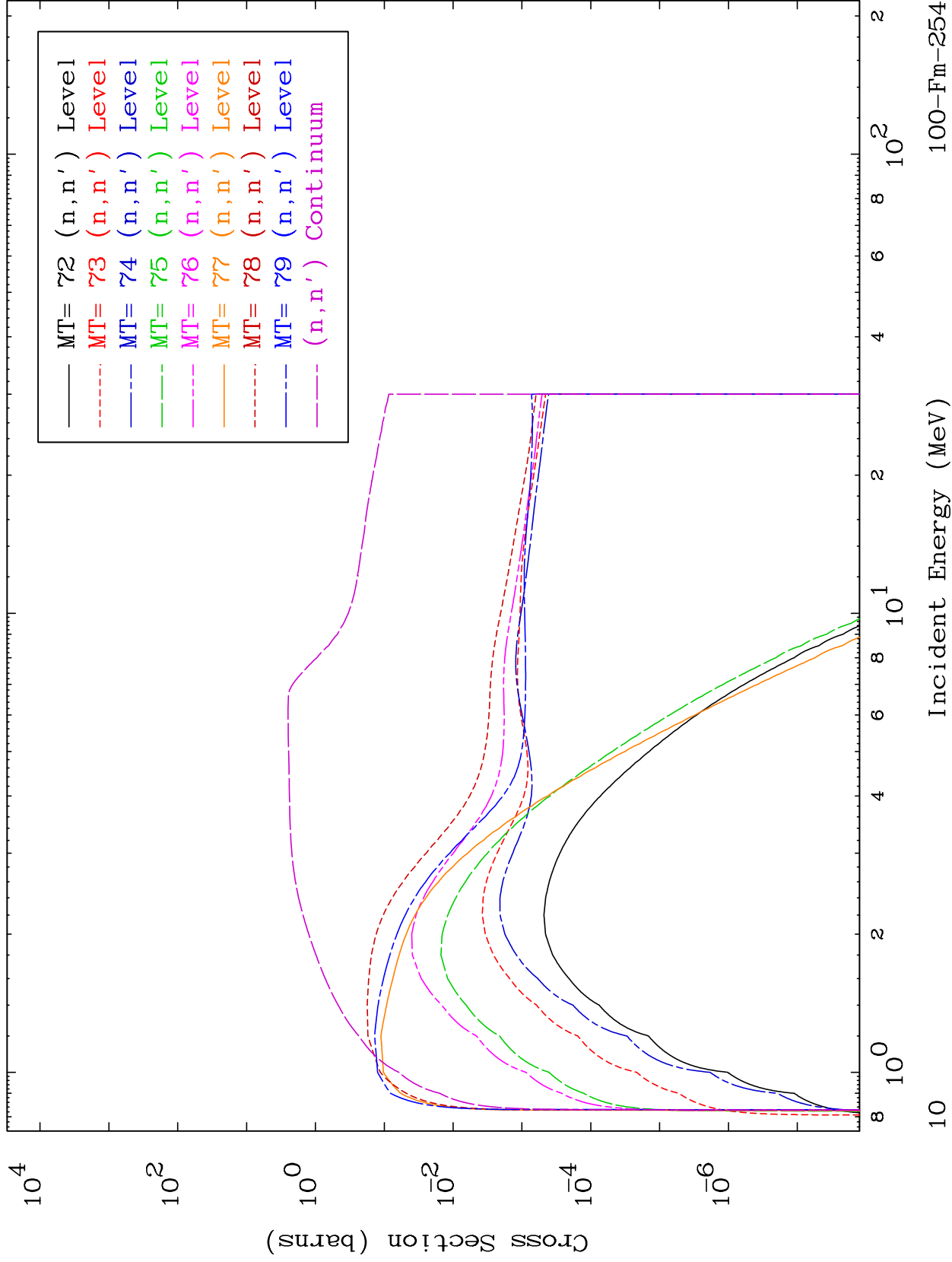
100-Fm-254

MAT 9935

(n,n') Level

100-Fm-254

293 Kelvin Cross Sections



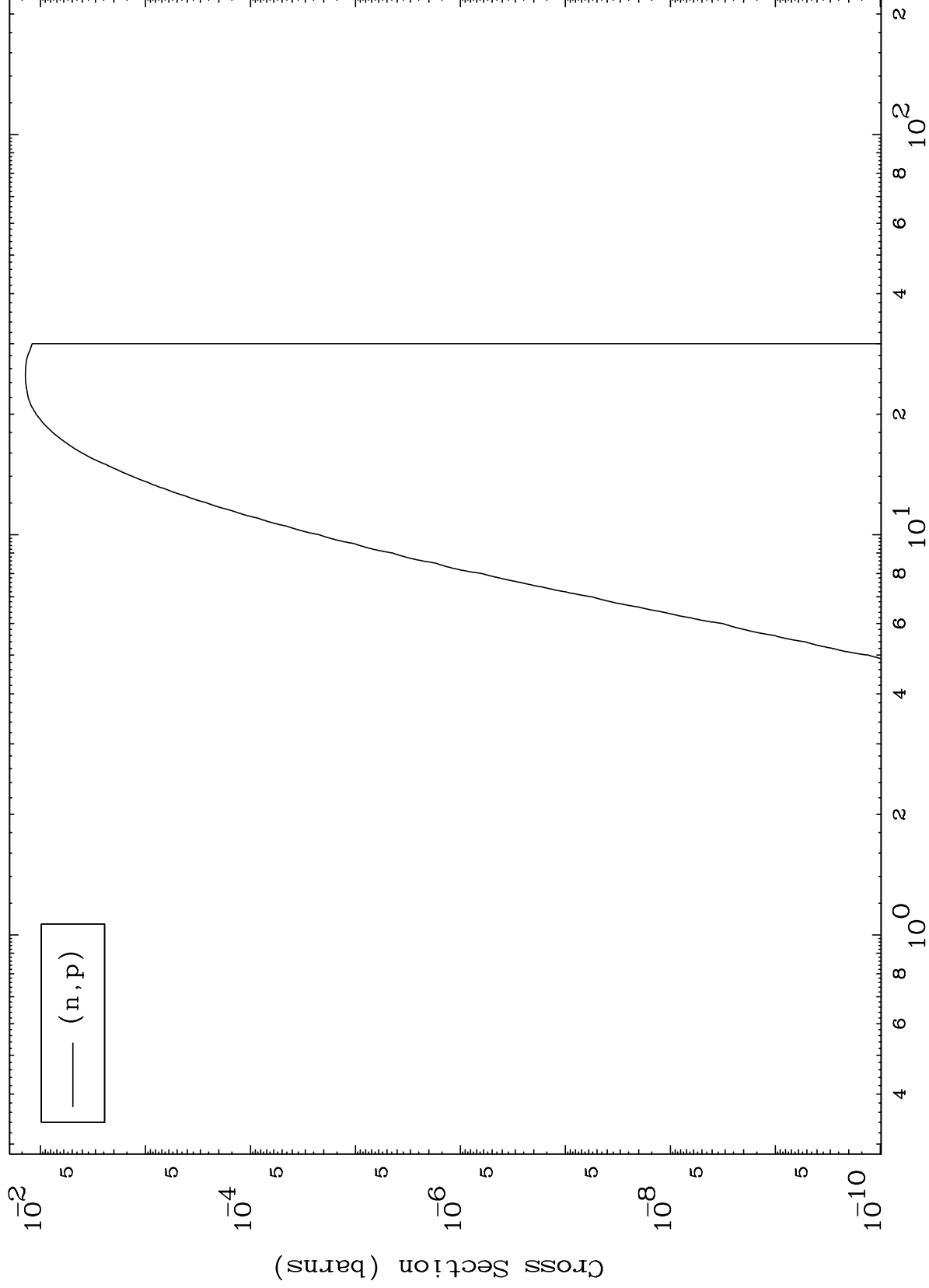
Incident Energy (MeV)

100-Fm-254

MAT 9935

(n,p) Levels
293 Kelvin Cross Sections

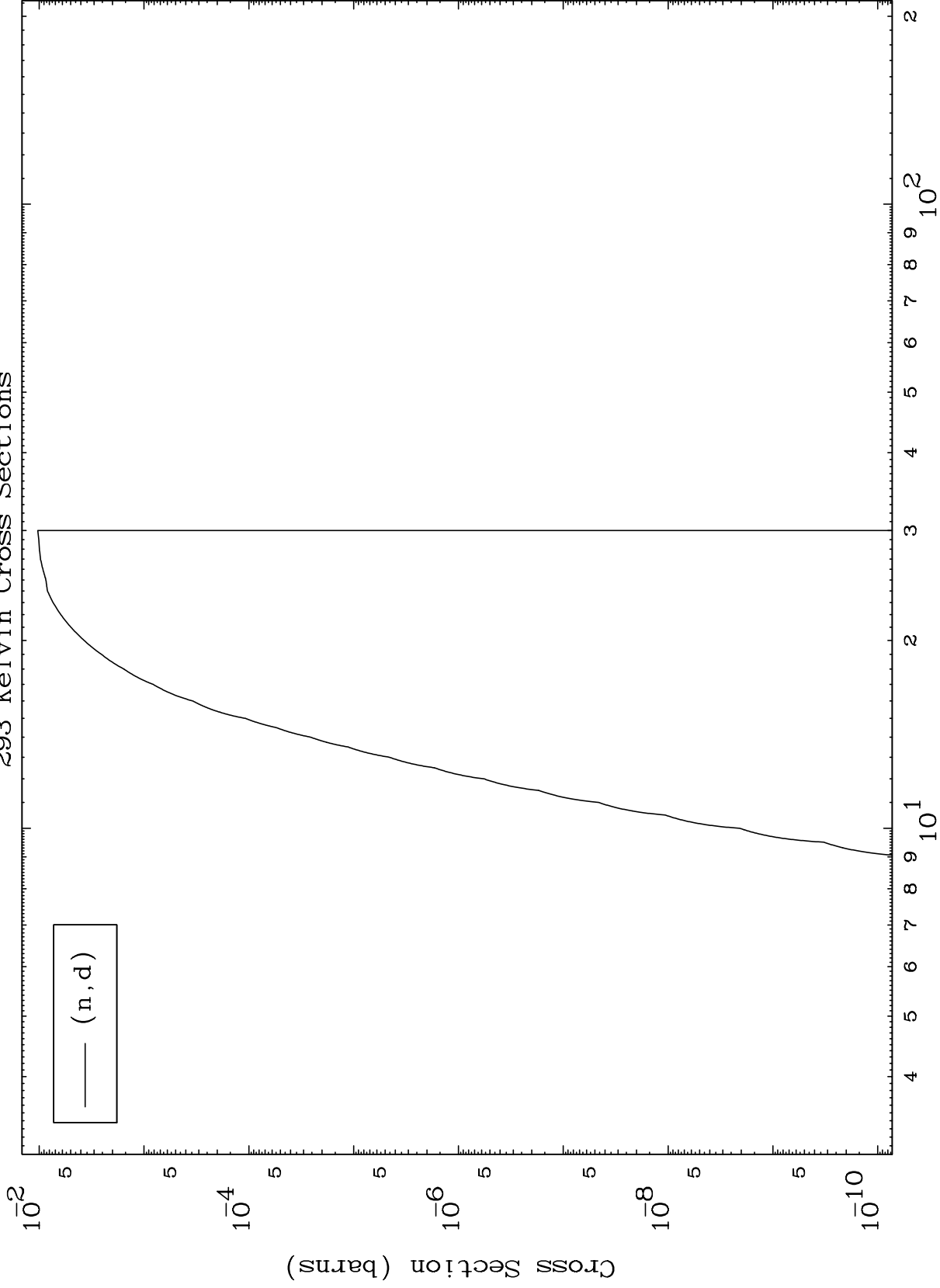
100-Fm-254



MAT 9935

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-254



12

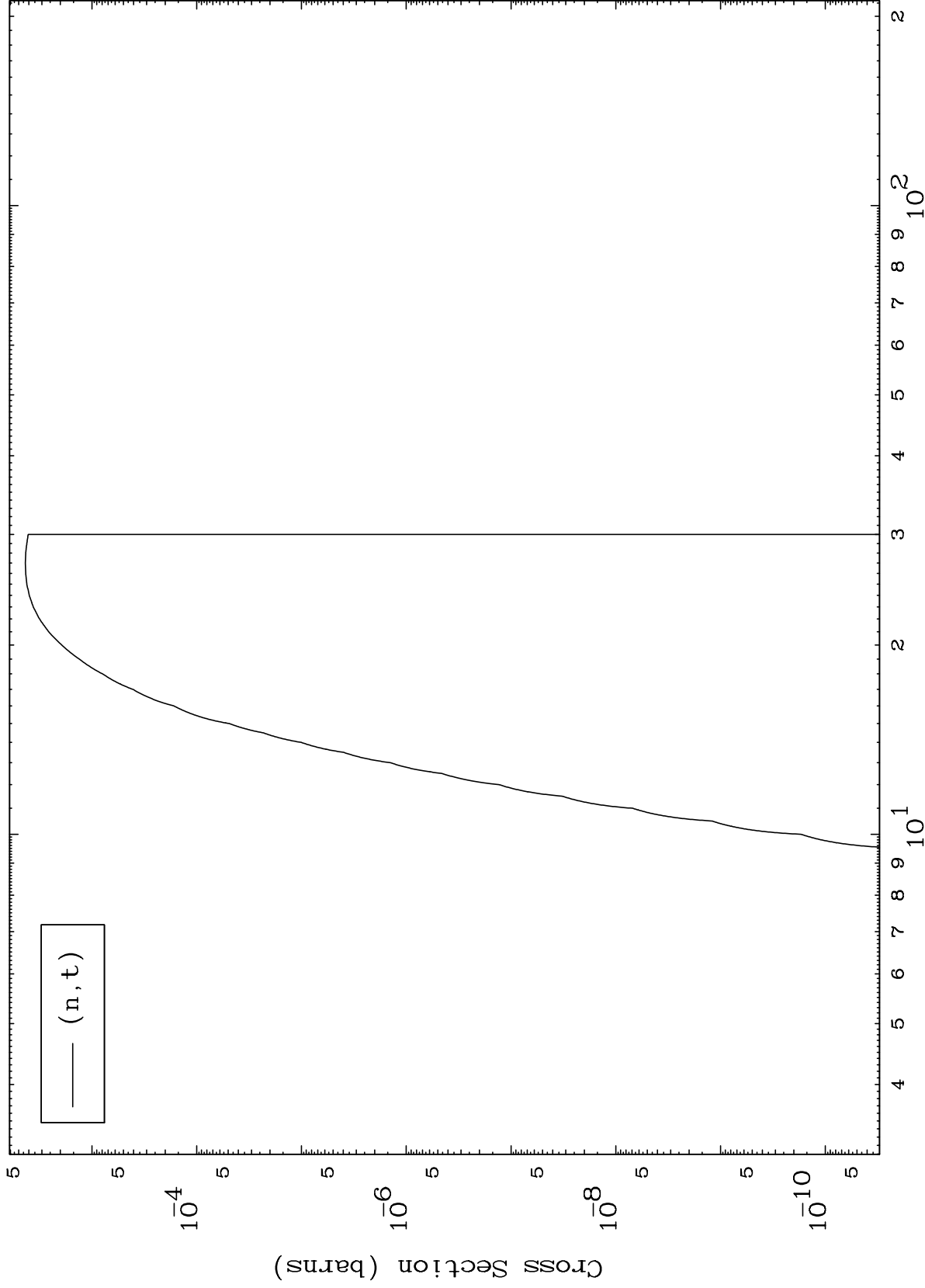
Incident Energy (MeV)

100-Fm-254

MAT 9935

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-254



13

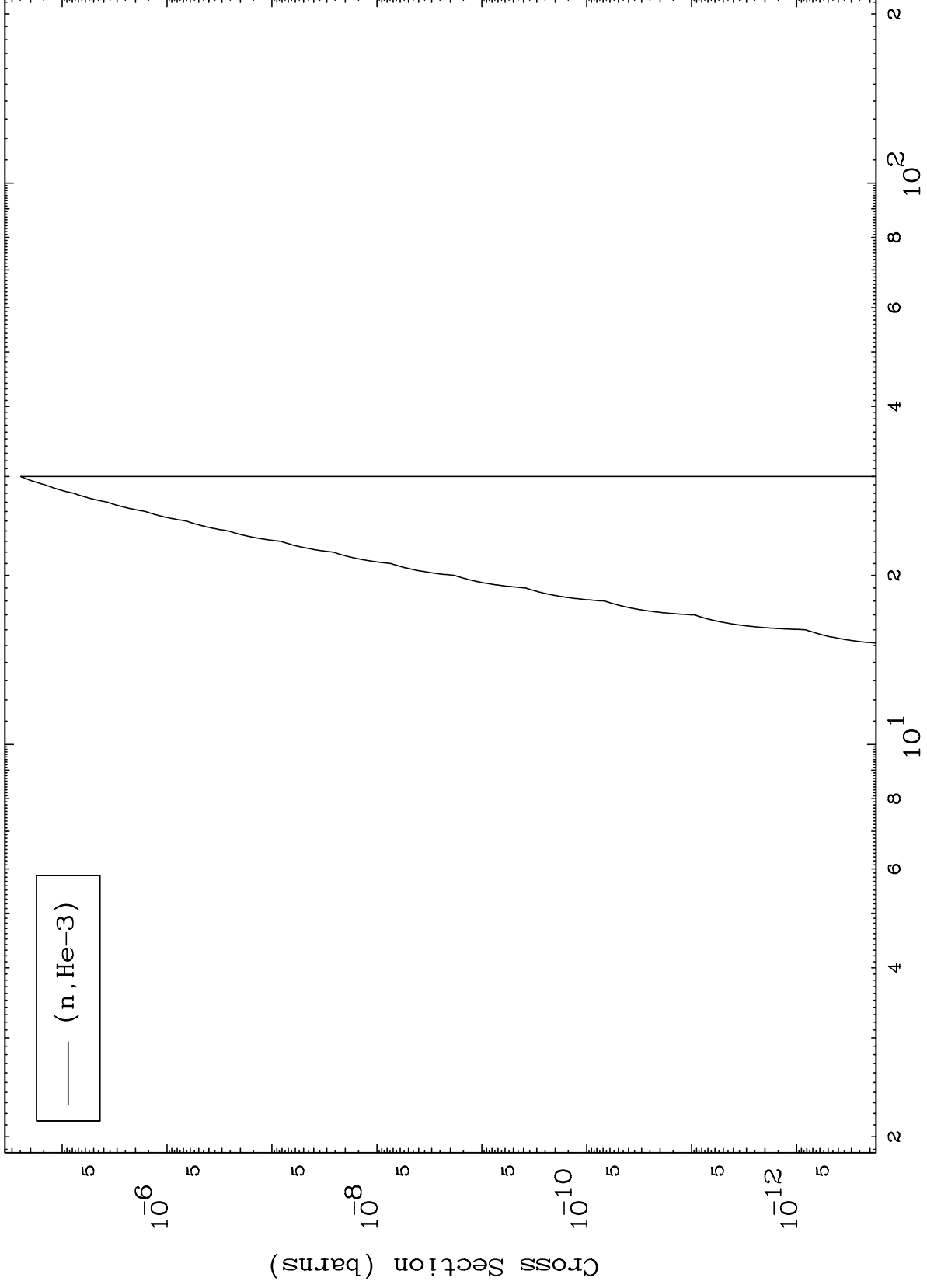
Incident Energy (MeV)

100-Fm-254

MAT 9935

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-254



14

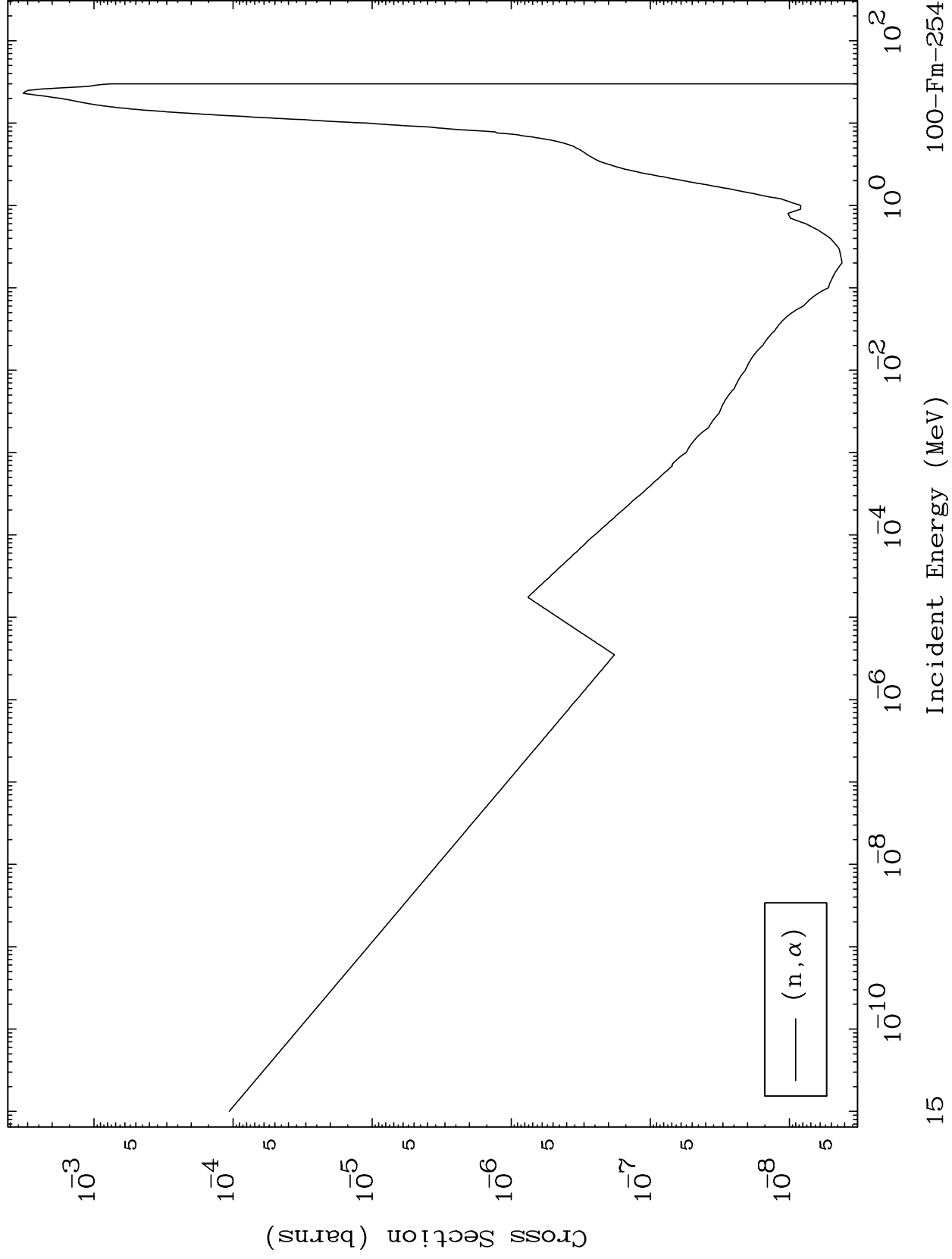
Incident Energy (MeV)

100-Fm-254

MAT 9935

(n, α) Levels
293 Kelvin Cross Sections

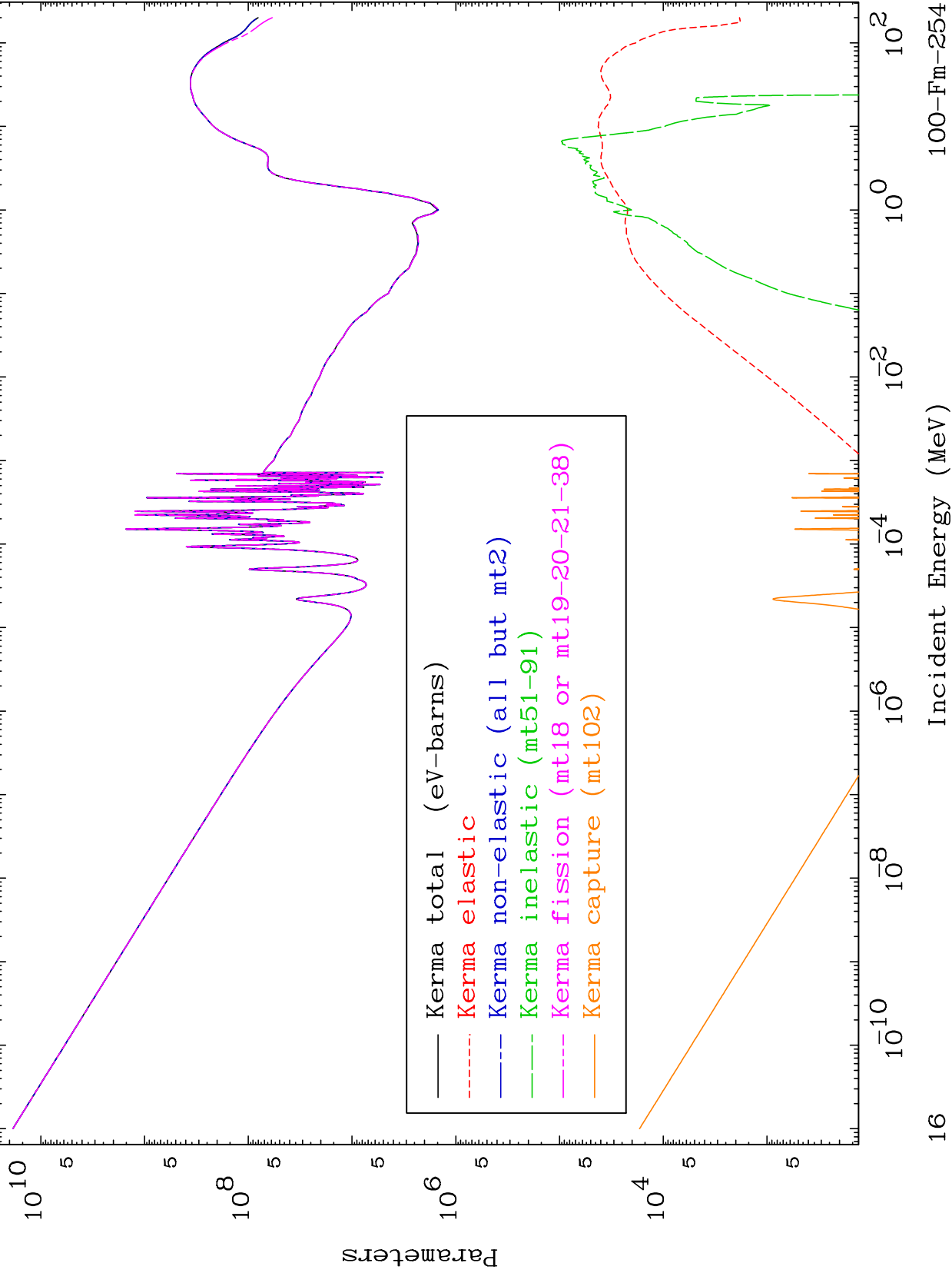
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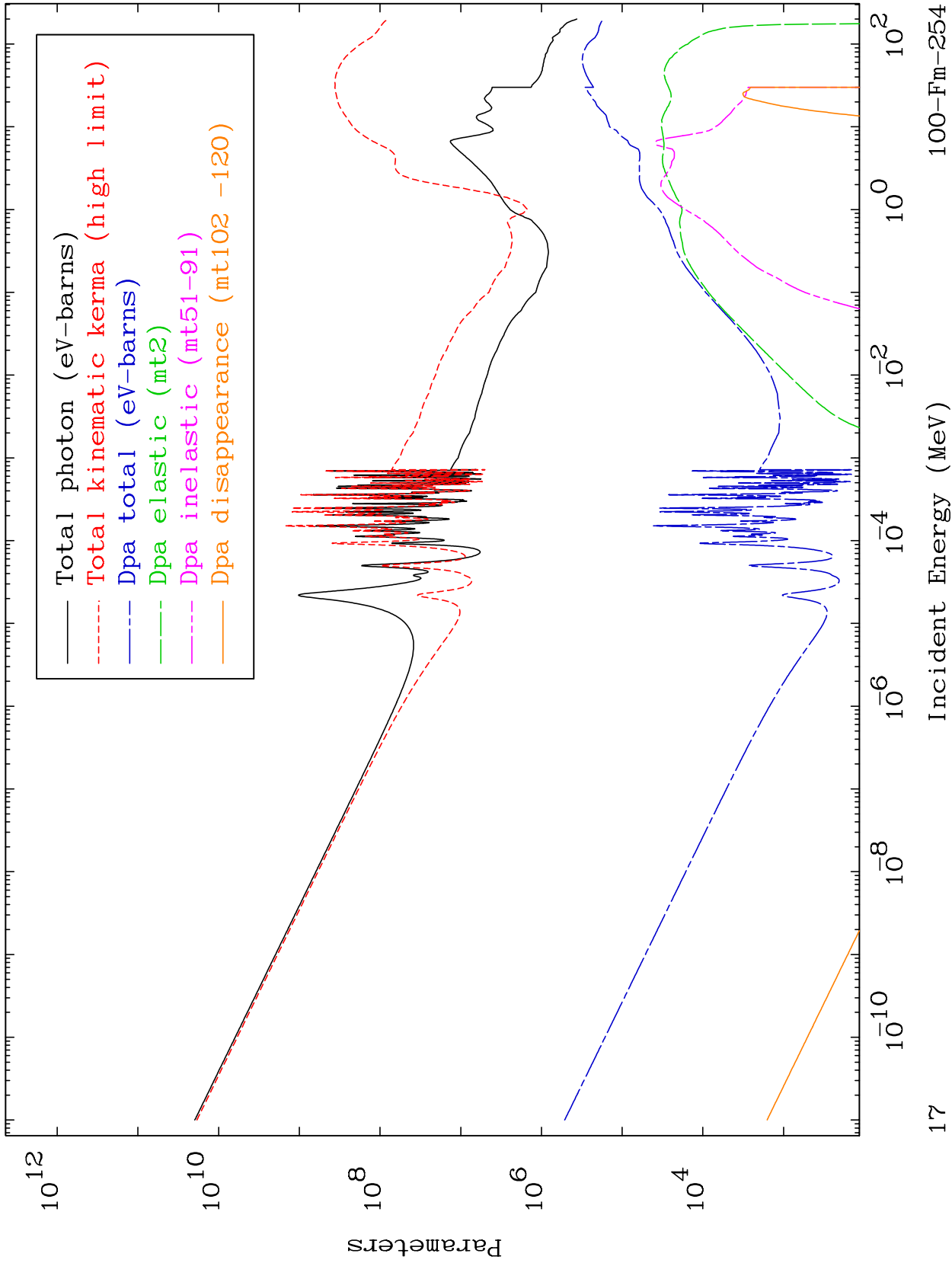


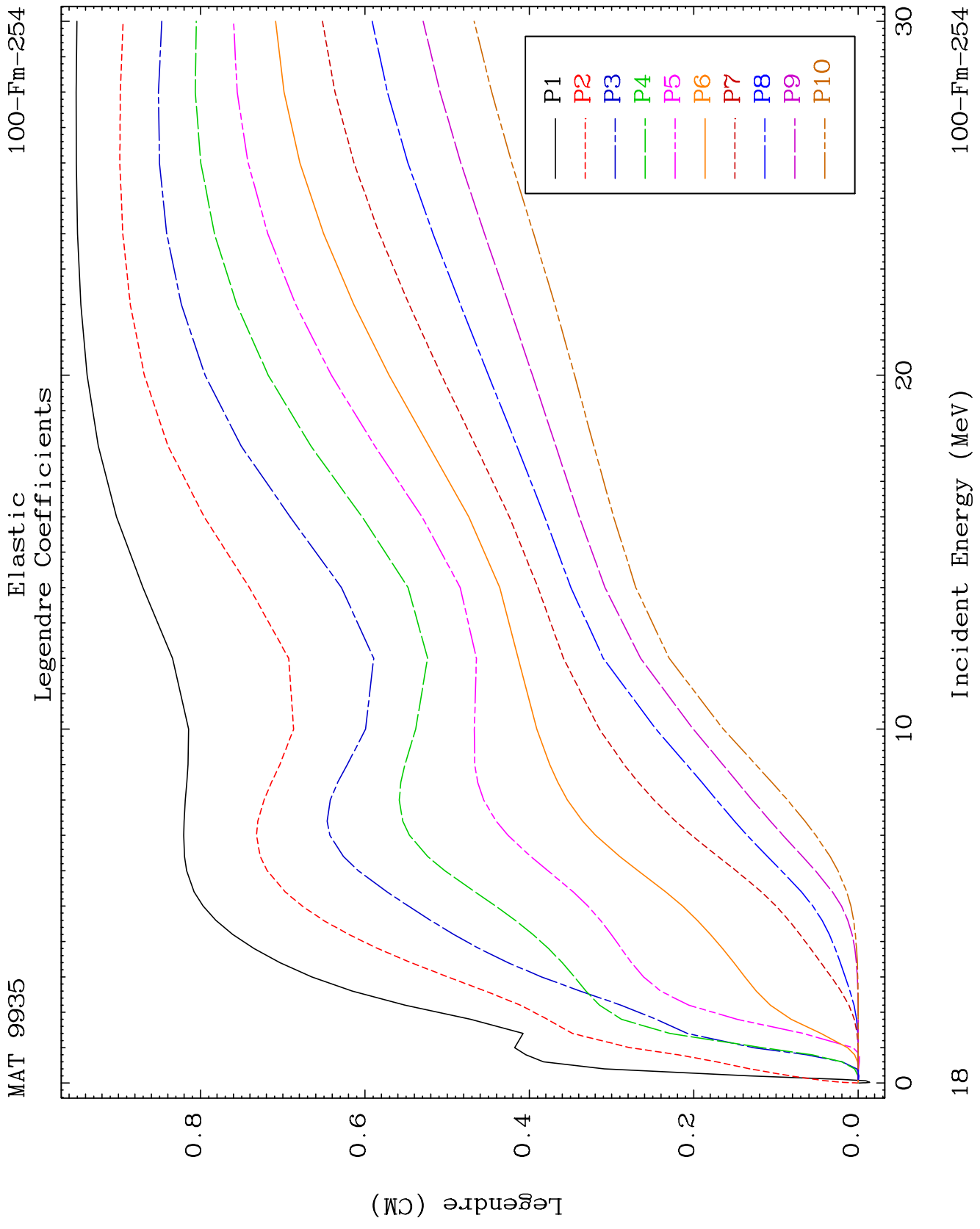
MAT 9935

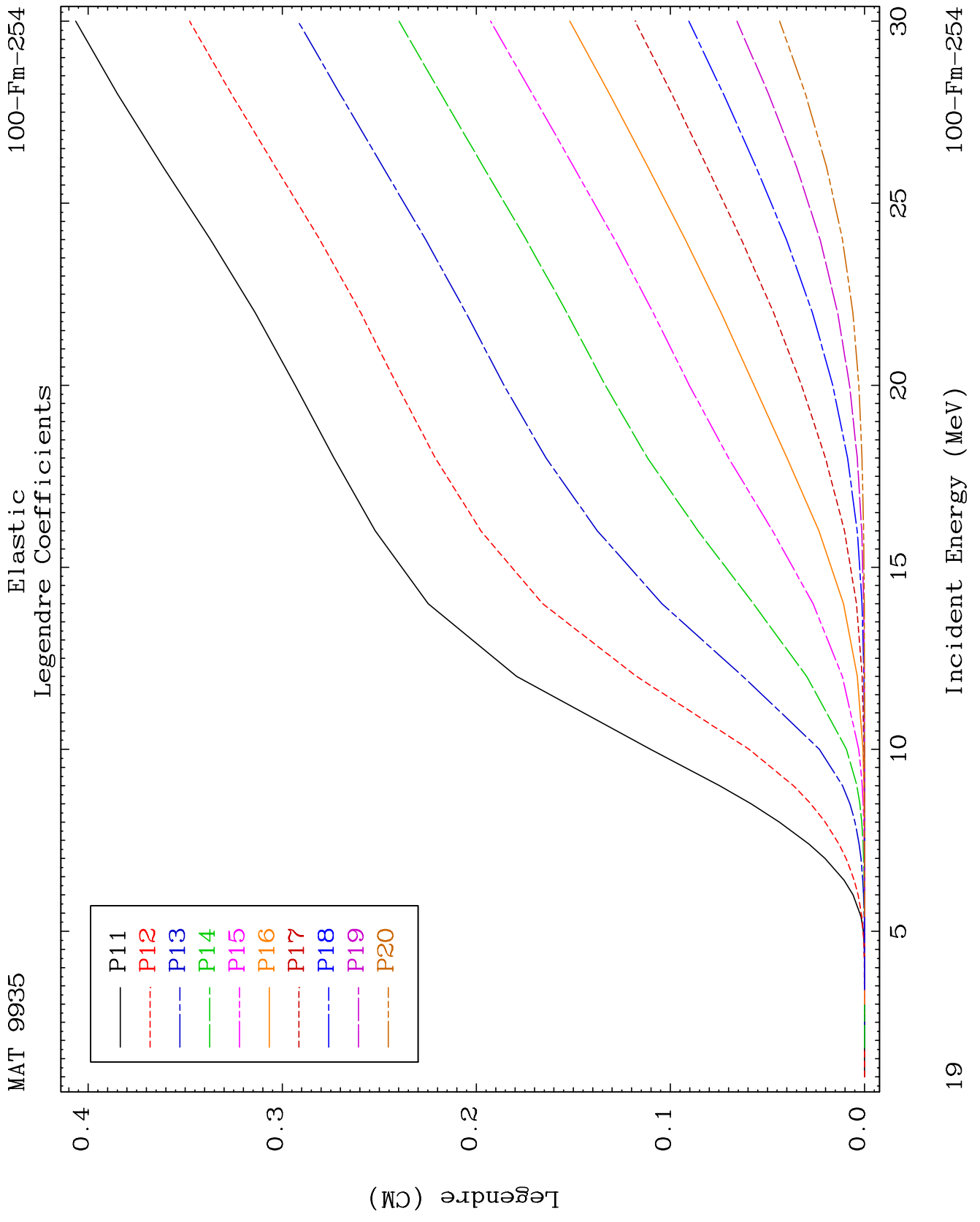
Energy Release
Parameters

100-Fm-254





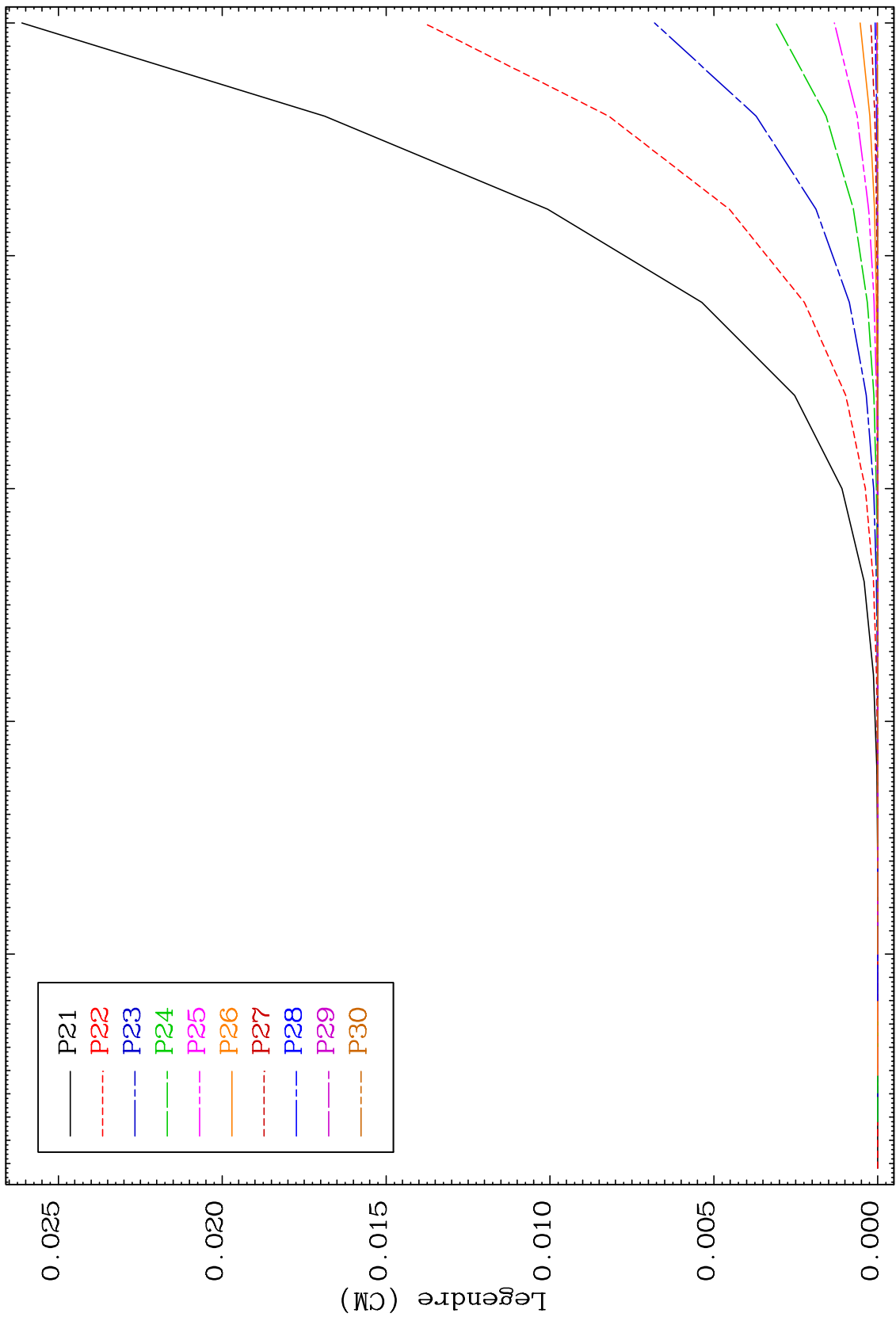




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Elastic
Legendre Coefficients

100-Fm-254



20

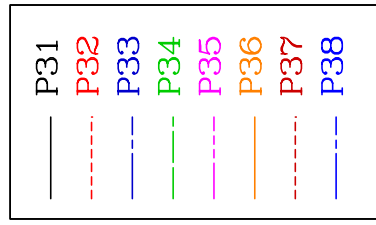
Incident Energy (MeV)

100-Fm-254

MAT 9935

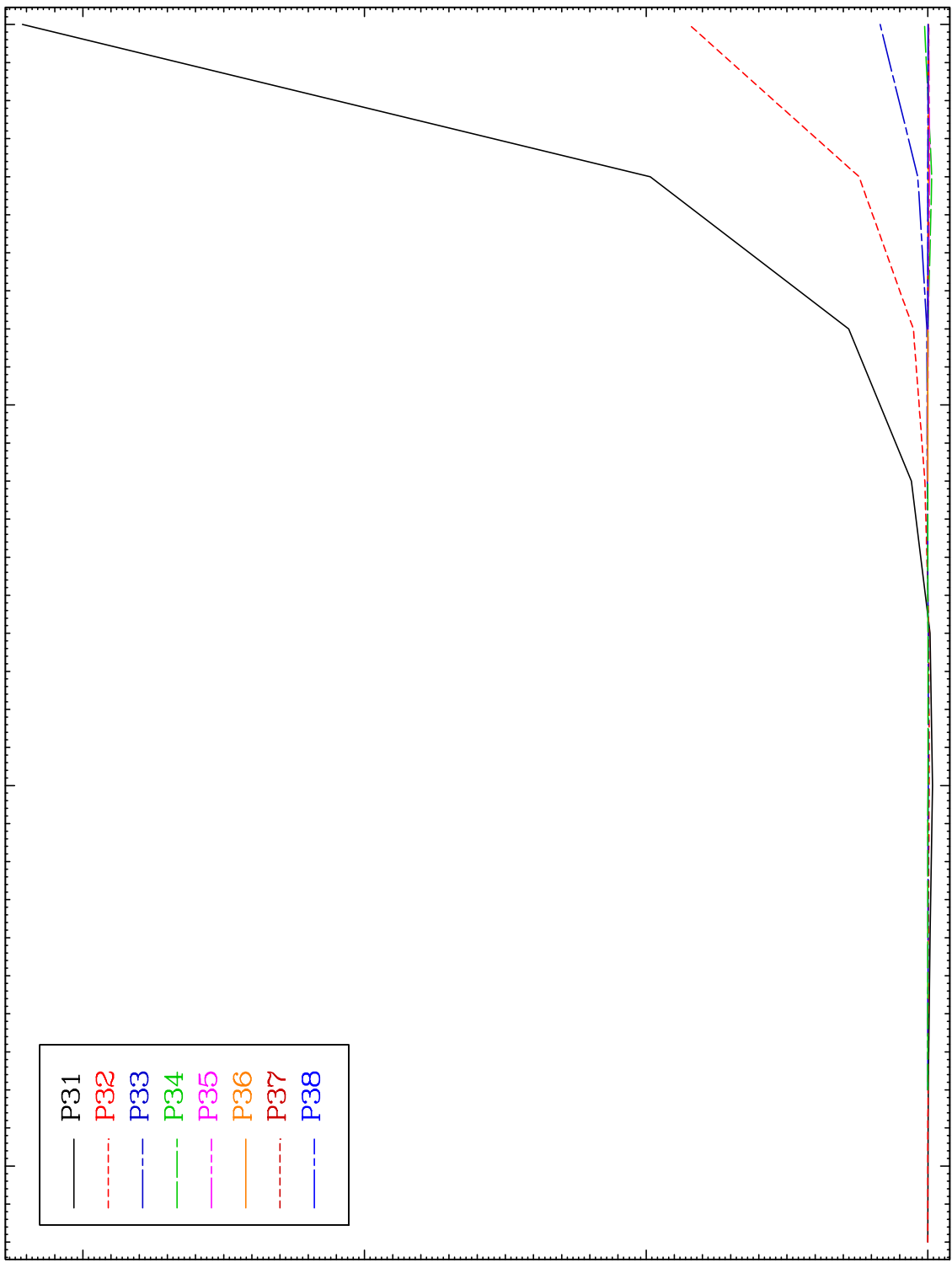
Elastic Legendre Coefficients

100-Fm-254



$\times 10^{-6}$

Legendre (CM)



30

25

20

15

21

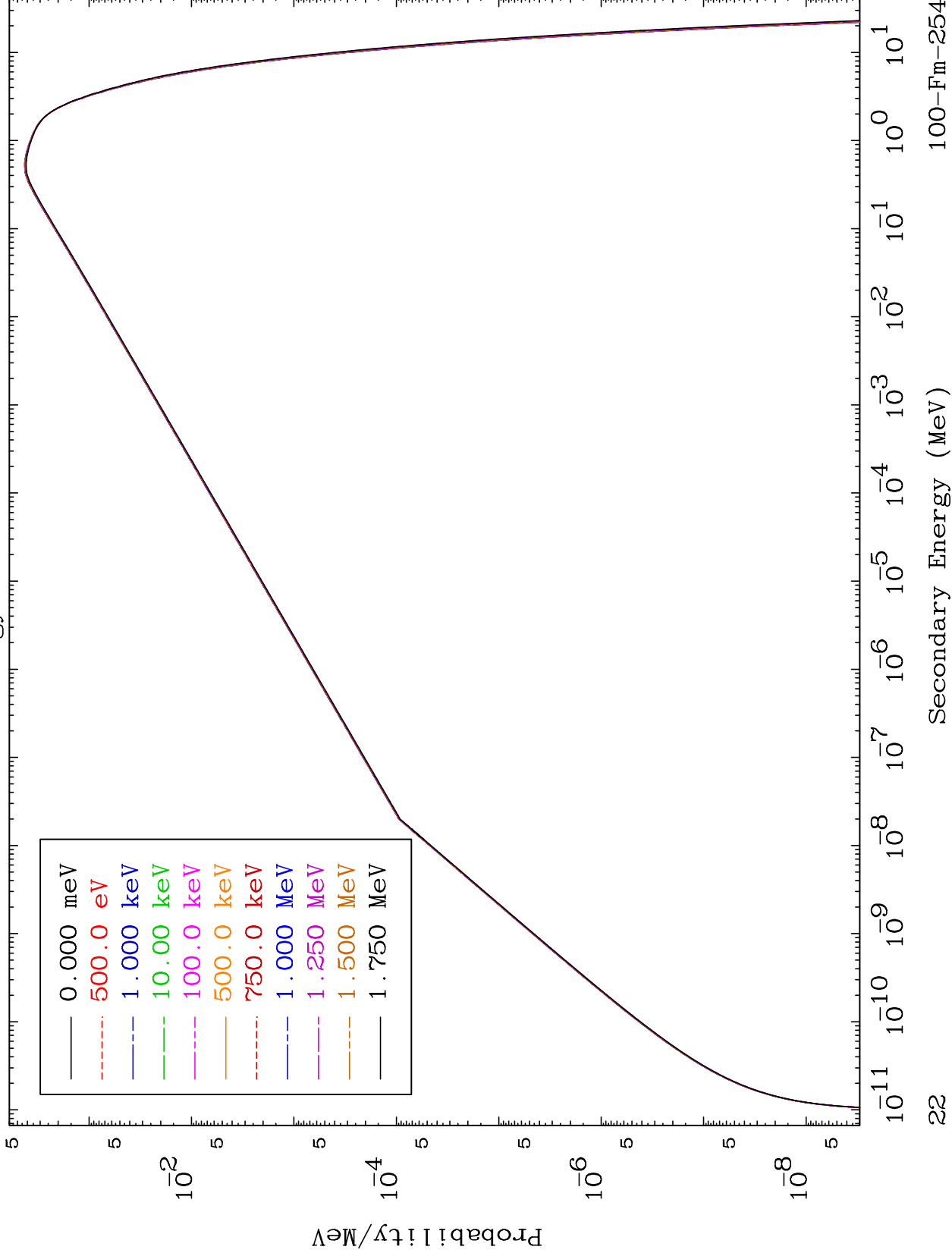
Incident Energy (MeV)

100-Fm-254

MAT 9935

Fission Energy Distribution

100-Fm-254

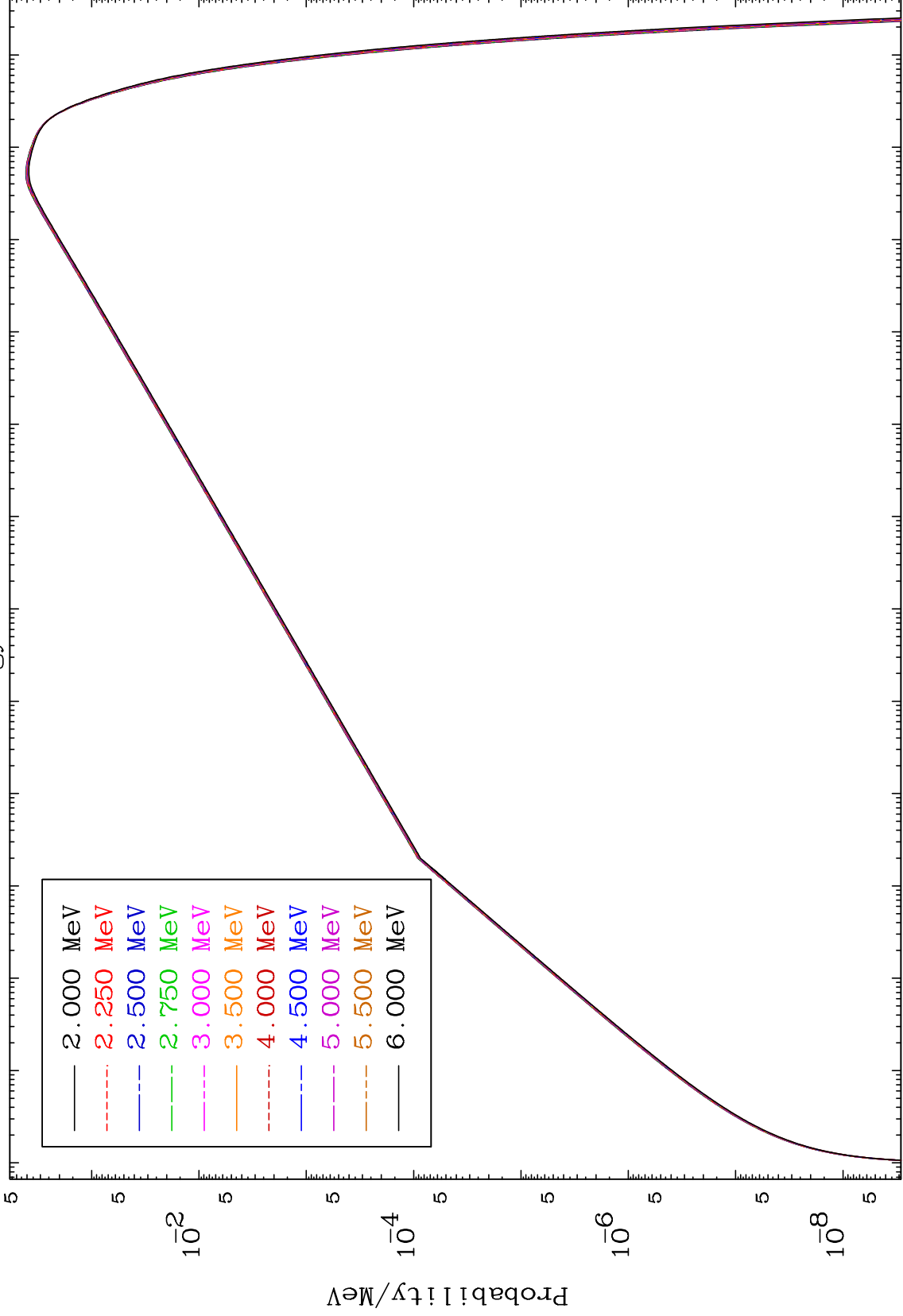


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Fission

100-Fm-254

Energy Distribution



23

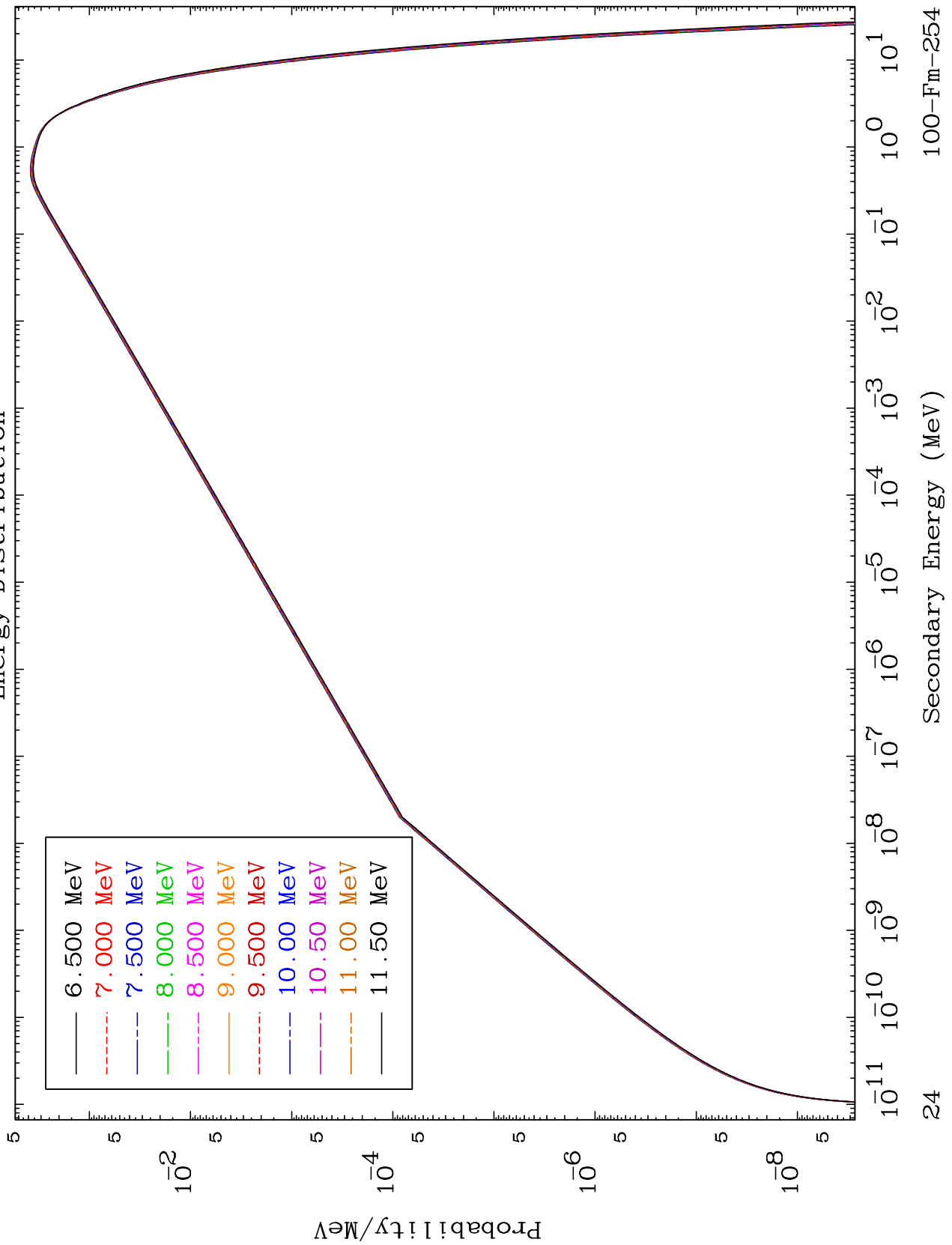
Secondary Energy (MeV)

100-Fm-254

MAT 9935

Fission Energy Distribution

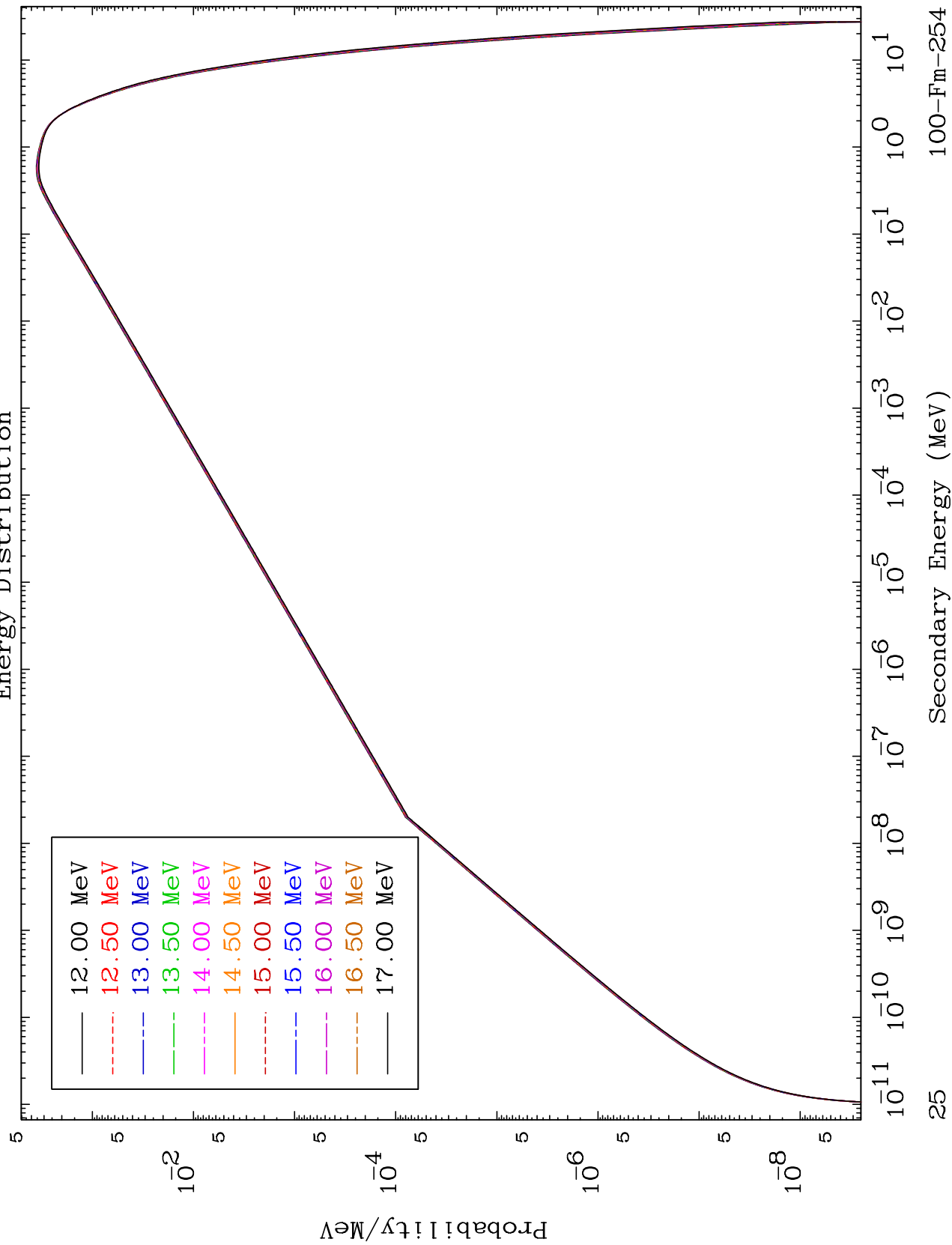
100-Fm-254



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Fission
Energy Distribution

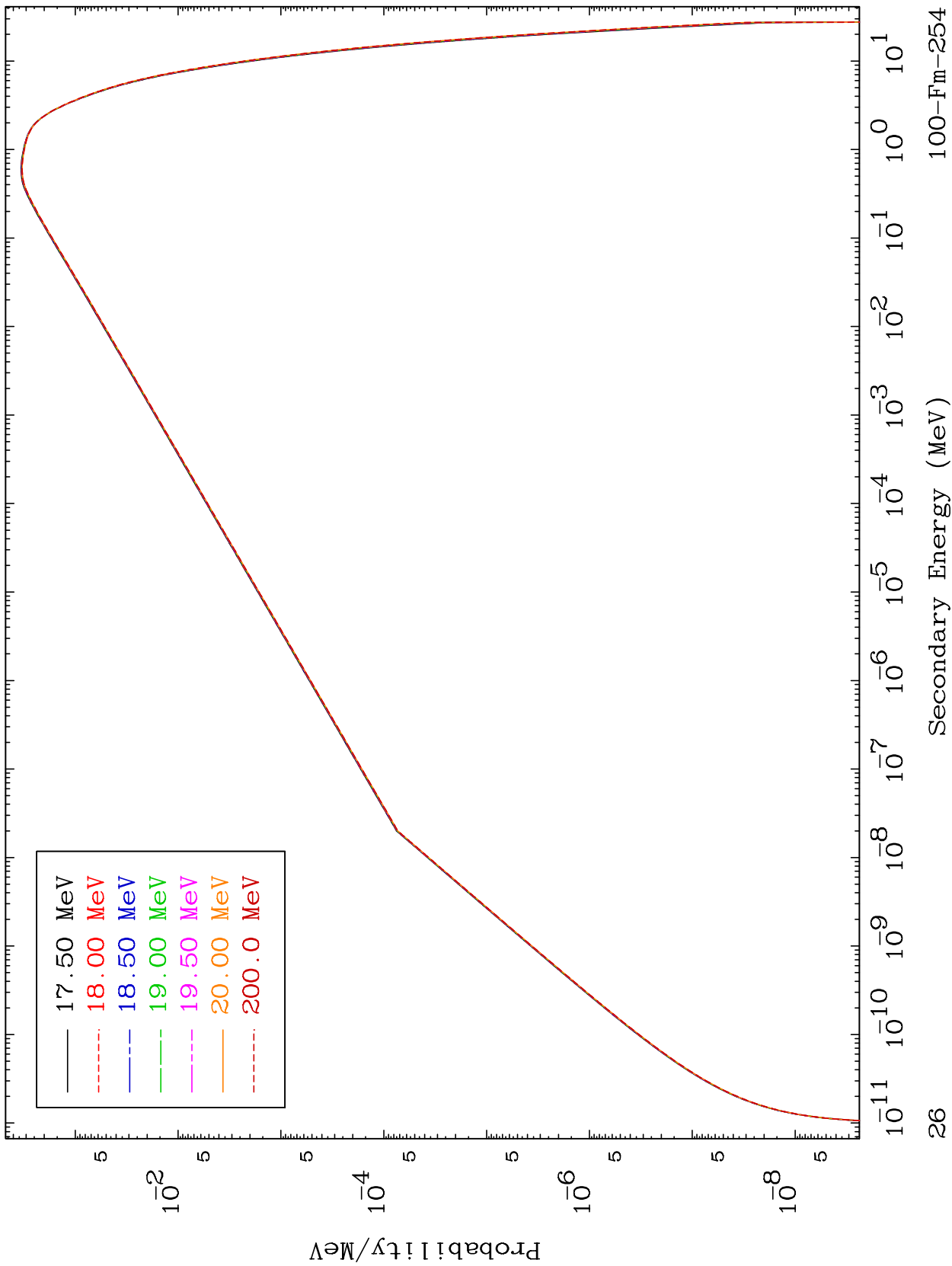
100-Fm-254



MAT 9935

Fission Energy Distribution

100-Fm-254

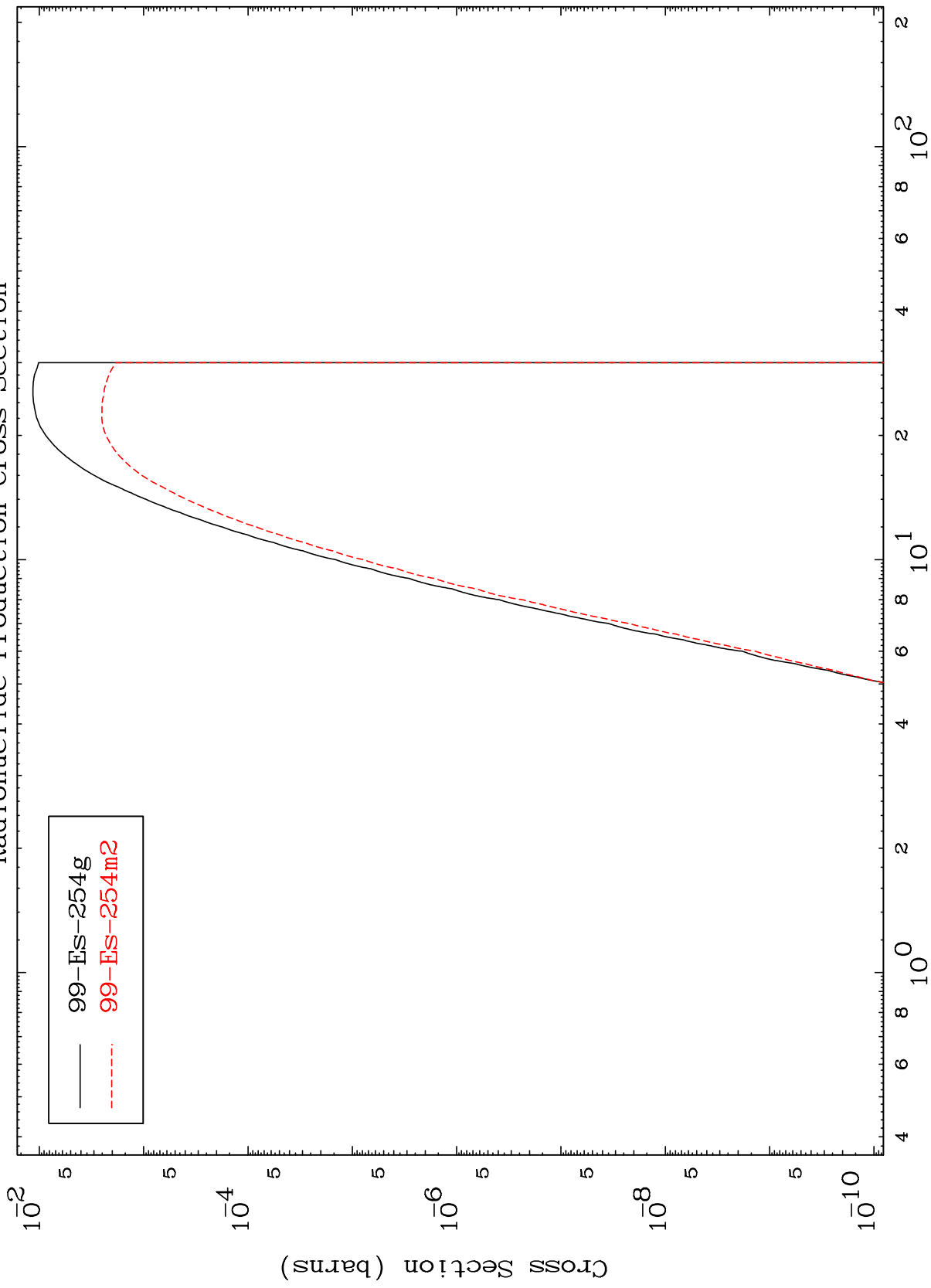


MAT 9935

(n,p)

100-Fm-254

Radionuclide Production Cross Section



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Incident Energy (MeV)

100-Fm-254