

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

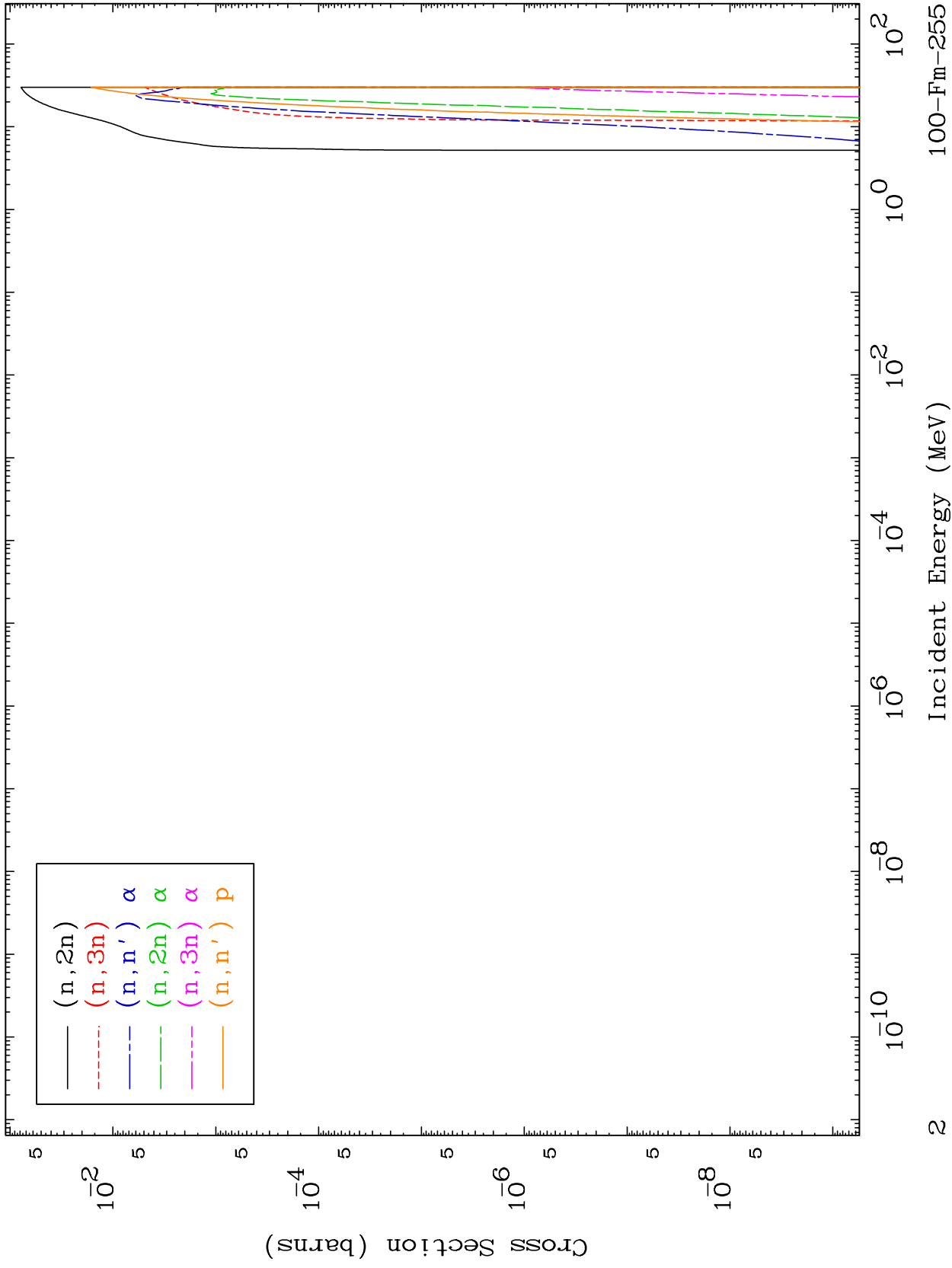
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9936

Neutron Production
293 Kelvin Cross Sections

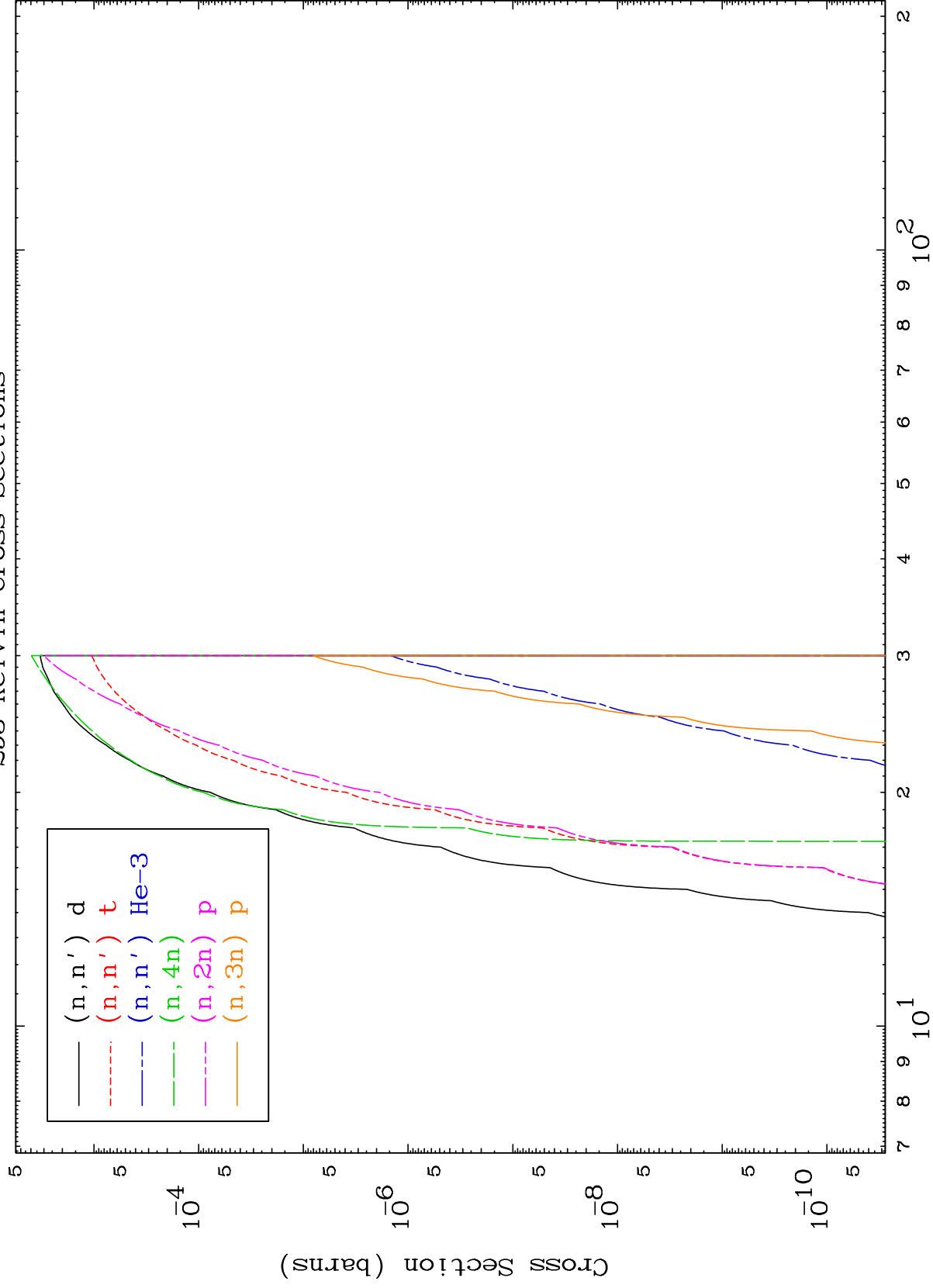
100-Fm-255



MAT 9936

Neutron Production
293 Kelvin Cross Sections

100-Fm-255



Incident Energy (MeV)

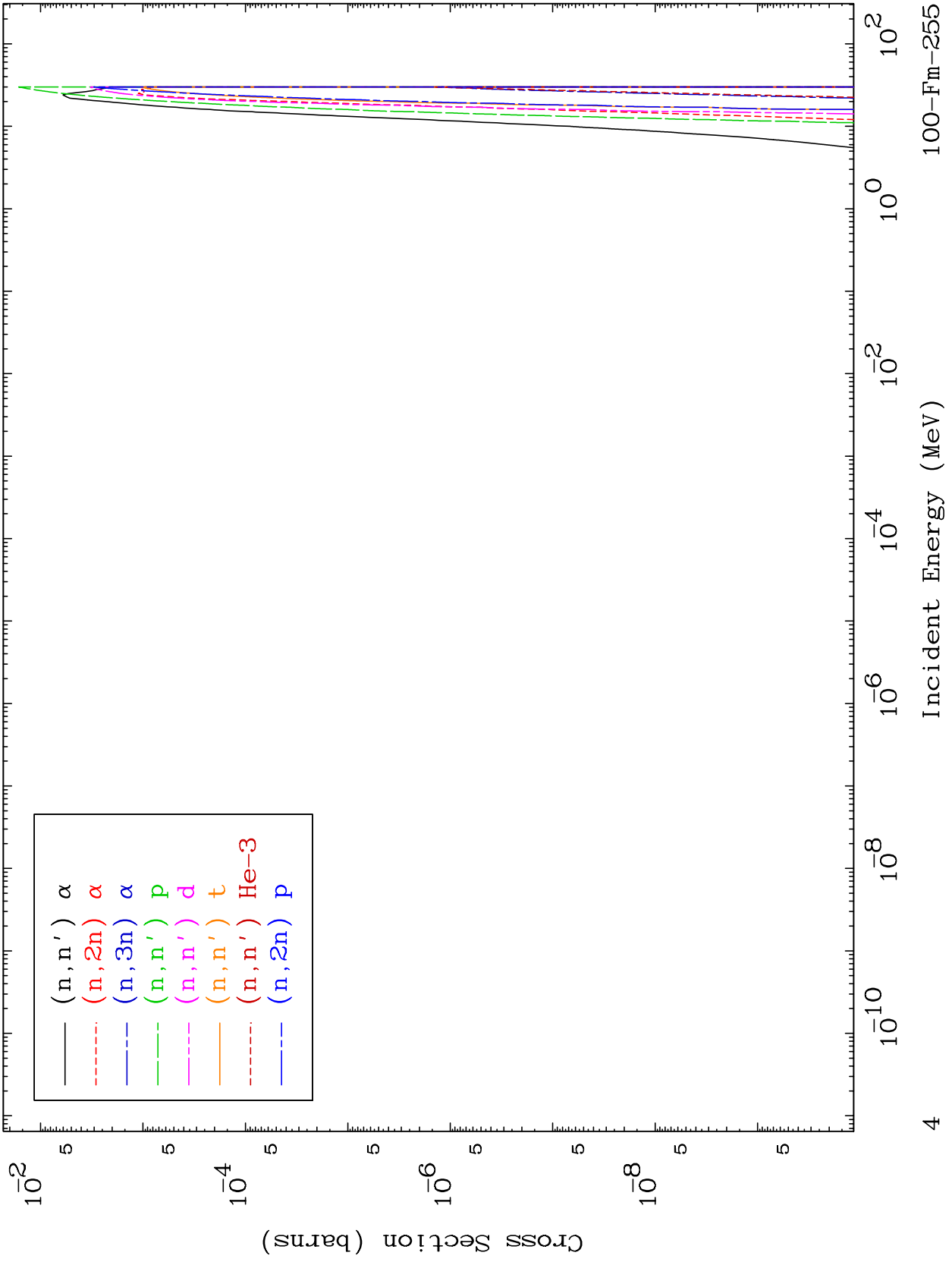
100-Fm-255

3

MAT 9936

Charged Particle
293 Kelvin Cross Sections

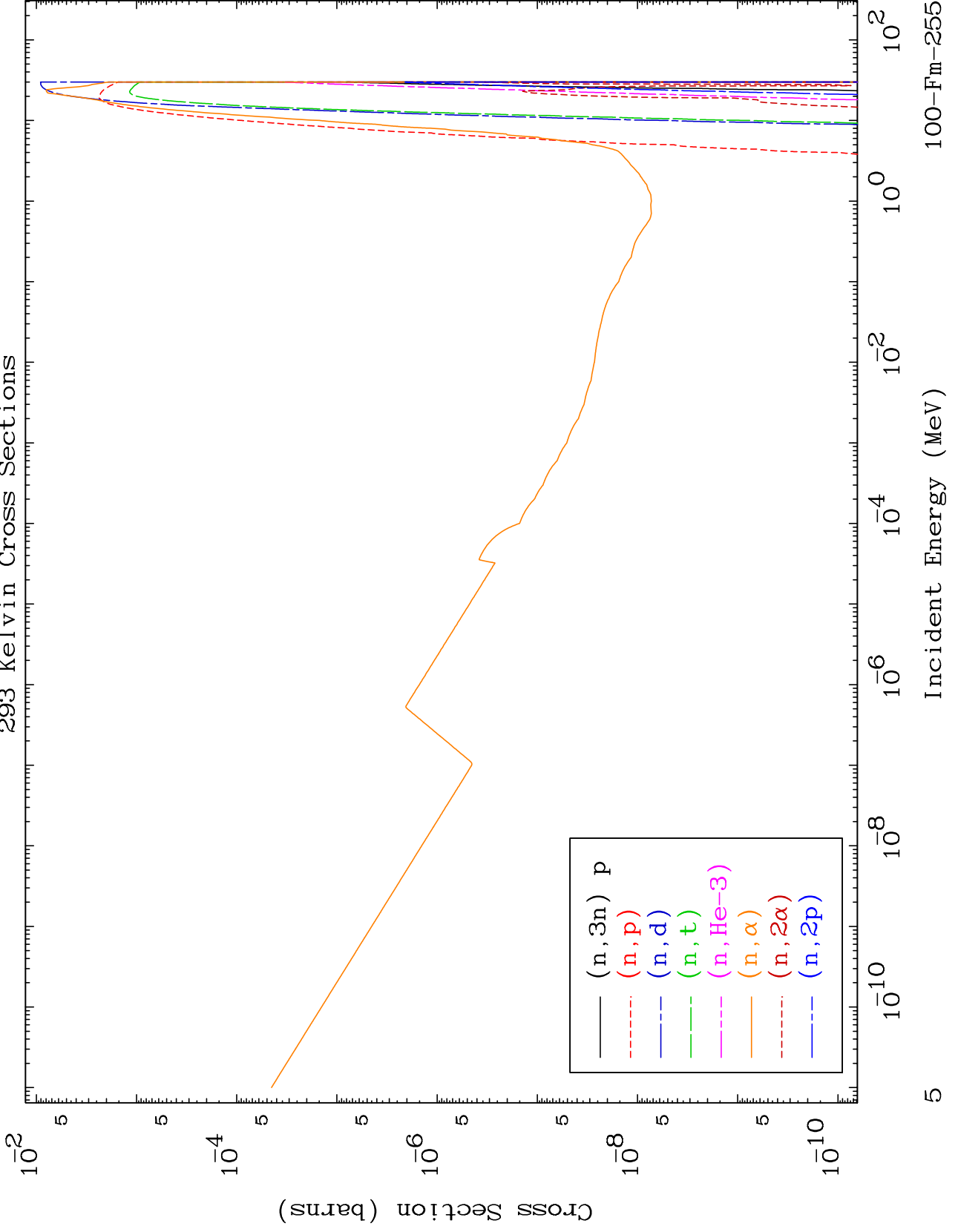
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MAT 9936

Charged Particle
293 Kelvin Cross Sections

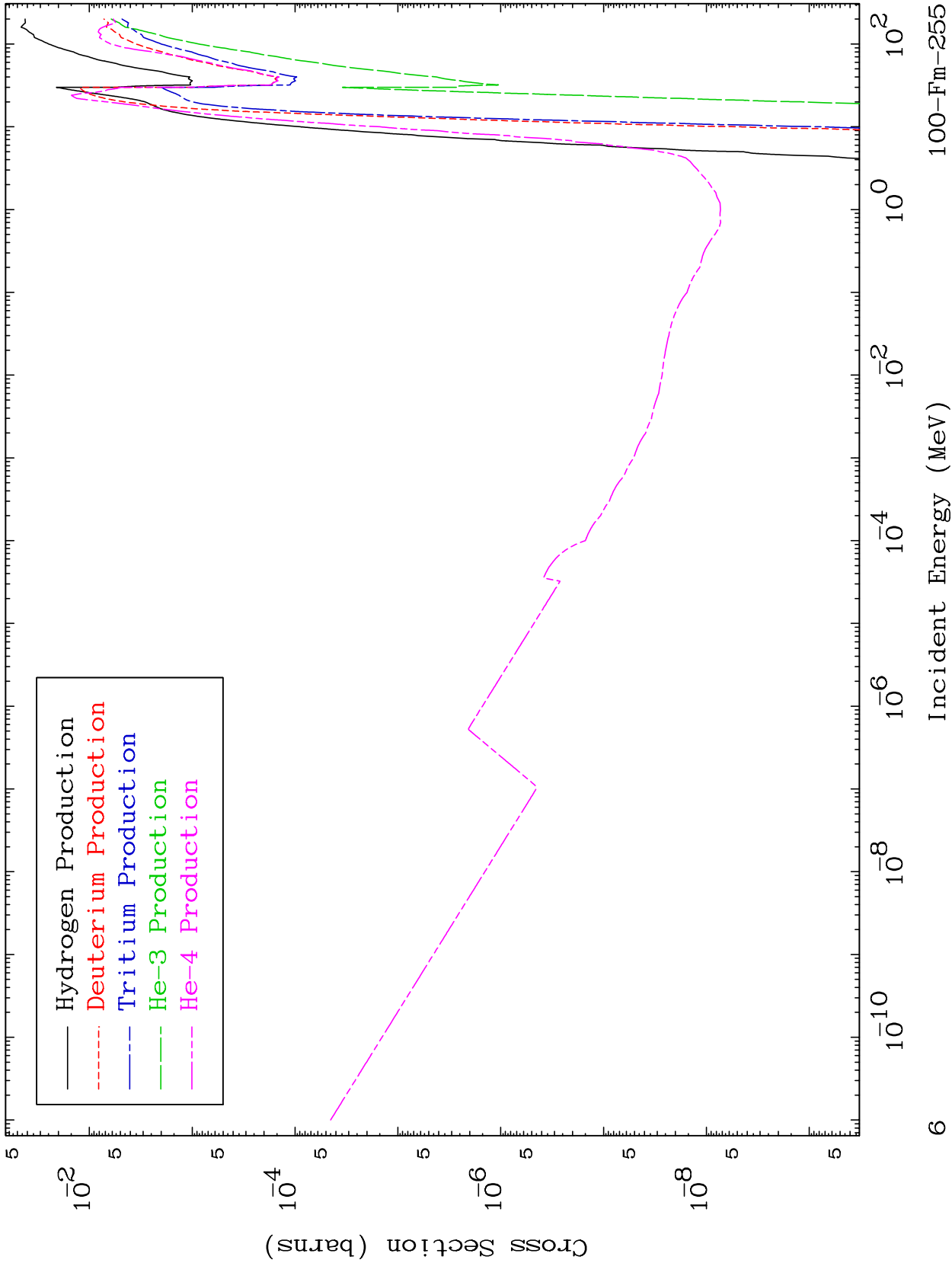
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MAT 9936

Particle Production
293 Kelvin Cross Sections

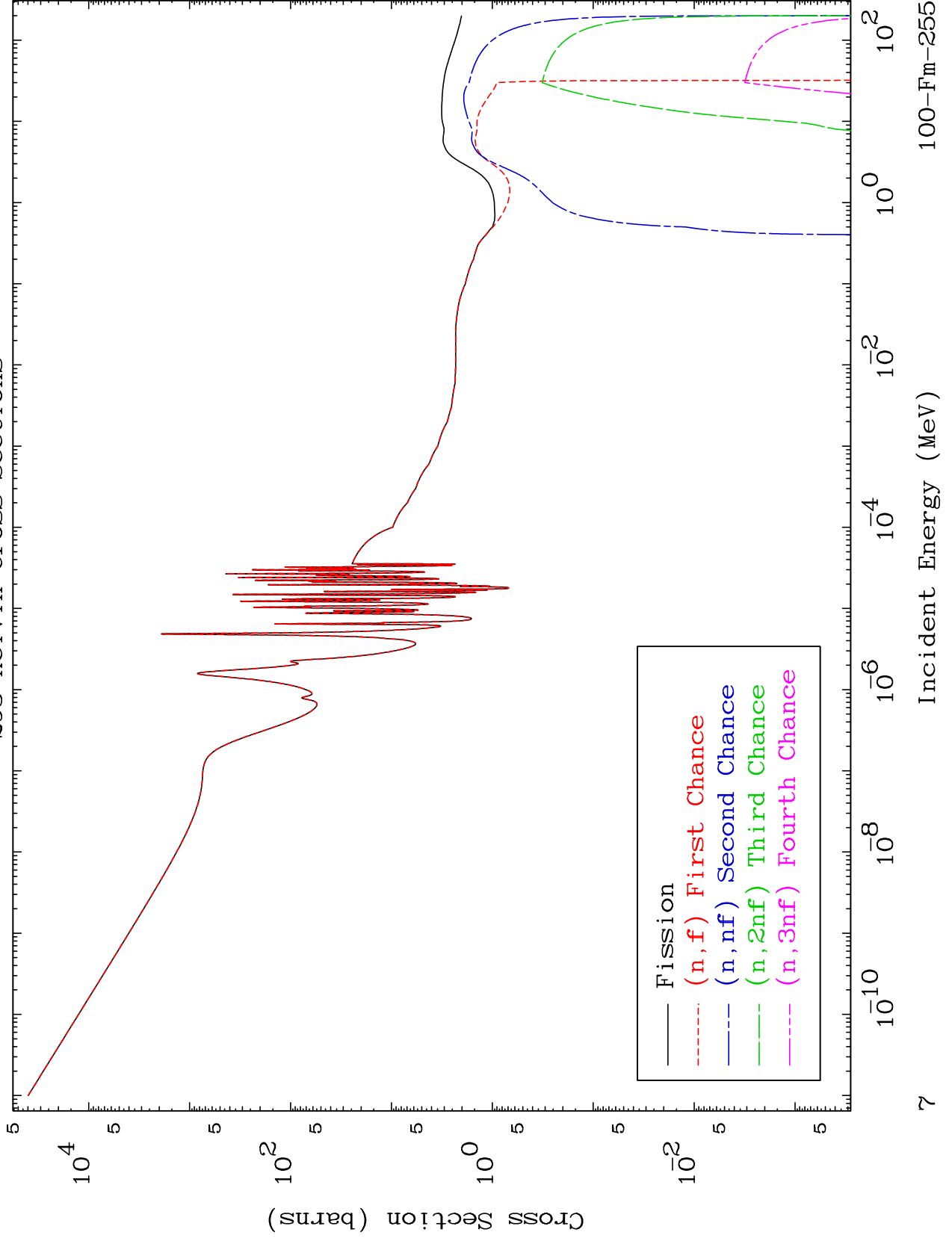
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MAT 9936

Fission
293 Kelvin Cross Sections

100-Fm-255

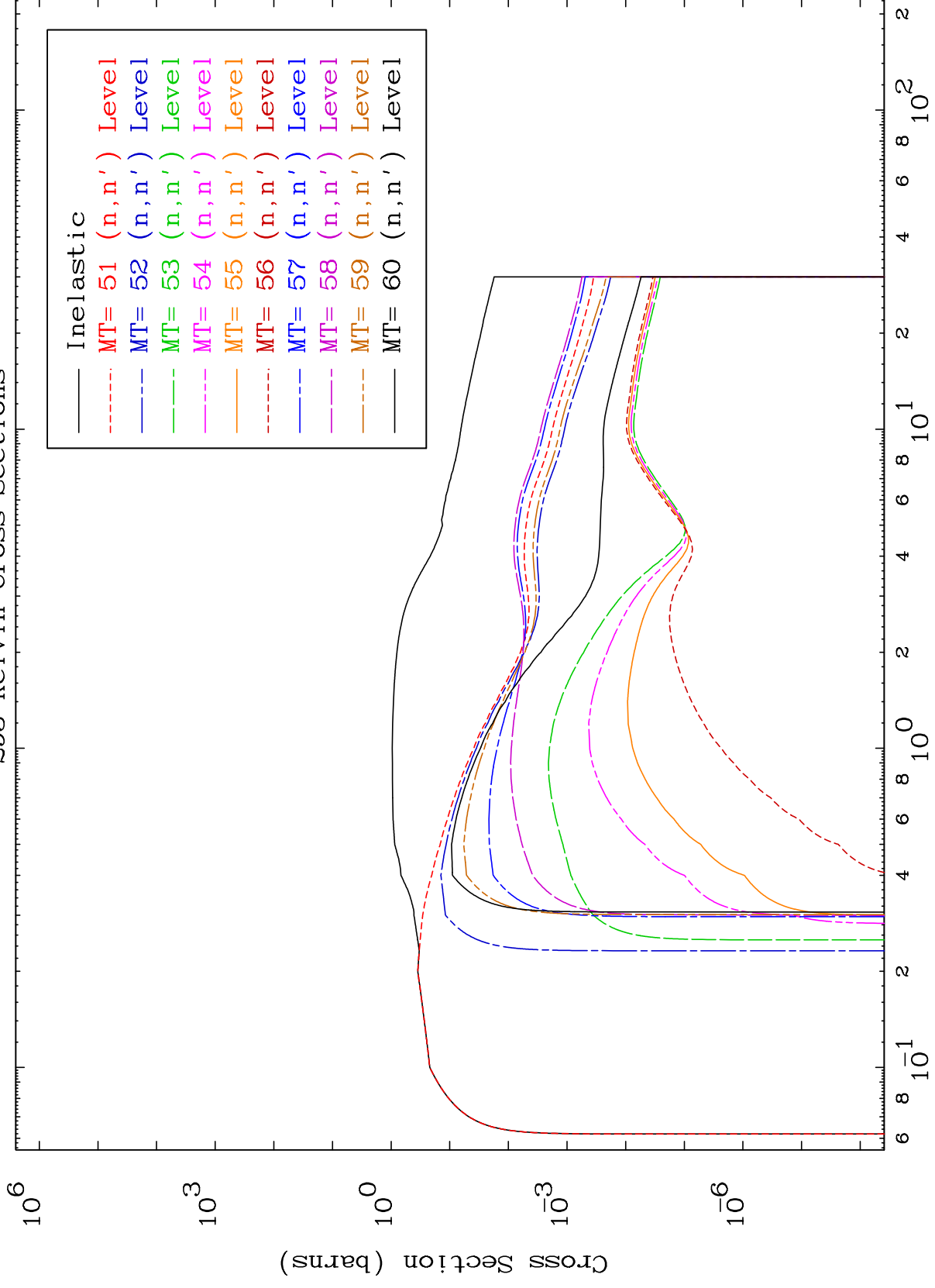


MAT 9936

(n,n') Level

100-Fm-255

293 Kelvin Cross Sections



8

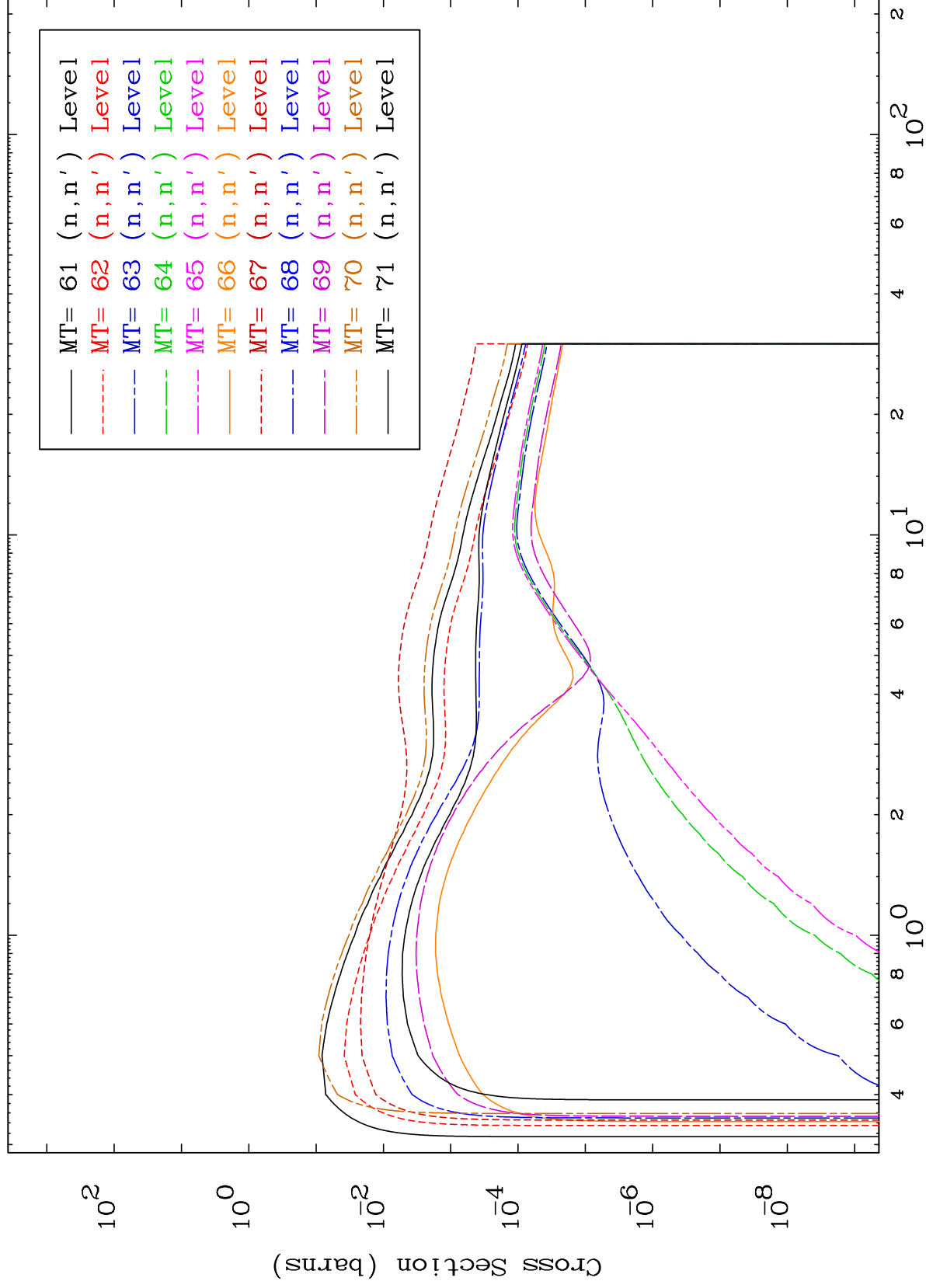
Incident Energy (MeV)

100-Fm-255

MAT 9936

(n,n') Level
293 Kelvin Cross Sections

100-Fm-255



9

Incident Energy (MeV)

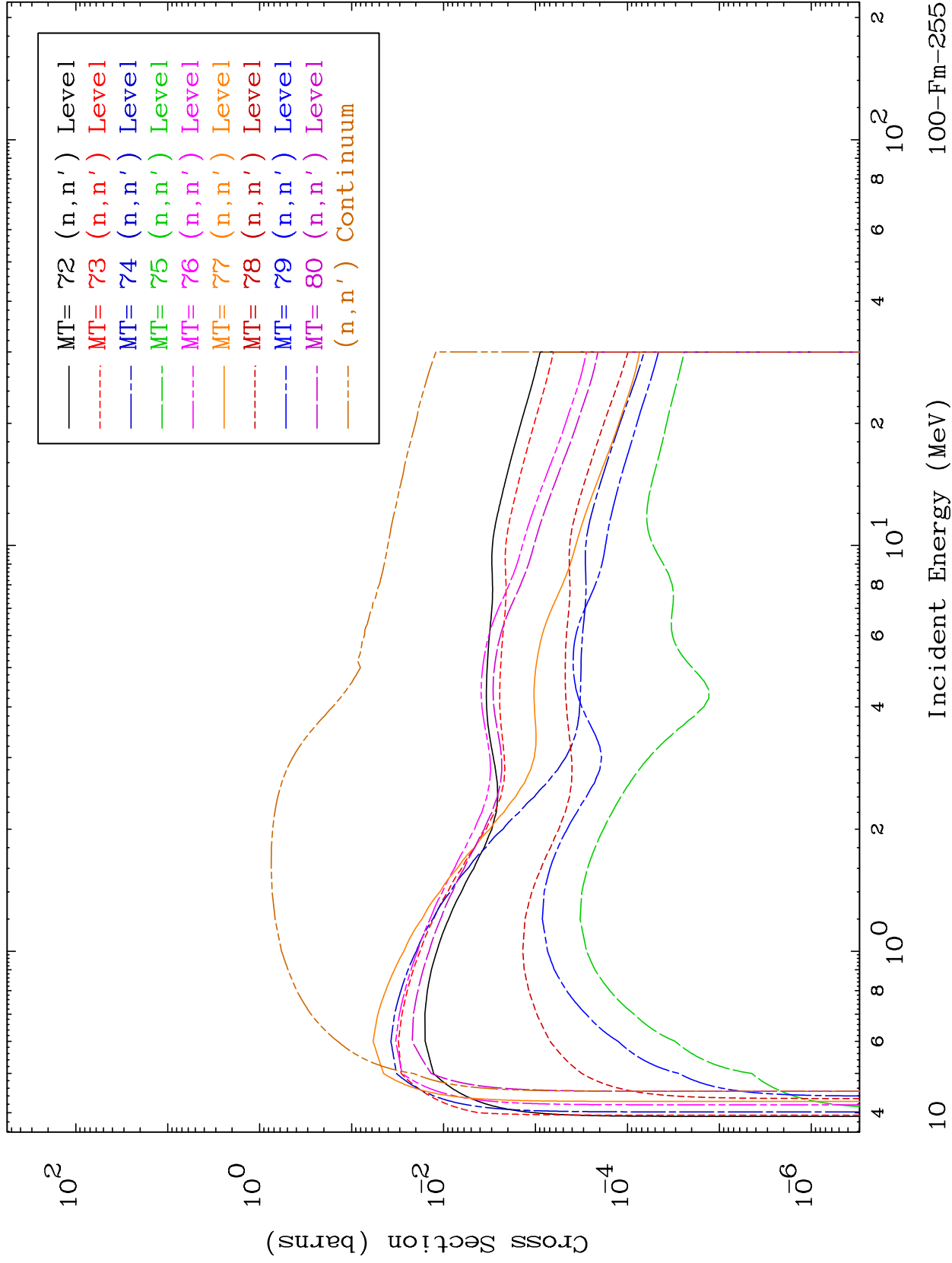
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MAT 9936

(n,n') Level

293 Kelvin Cross Sections

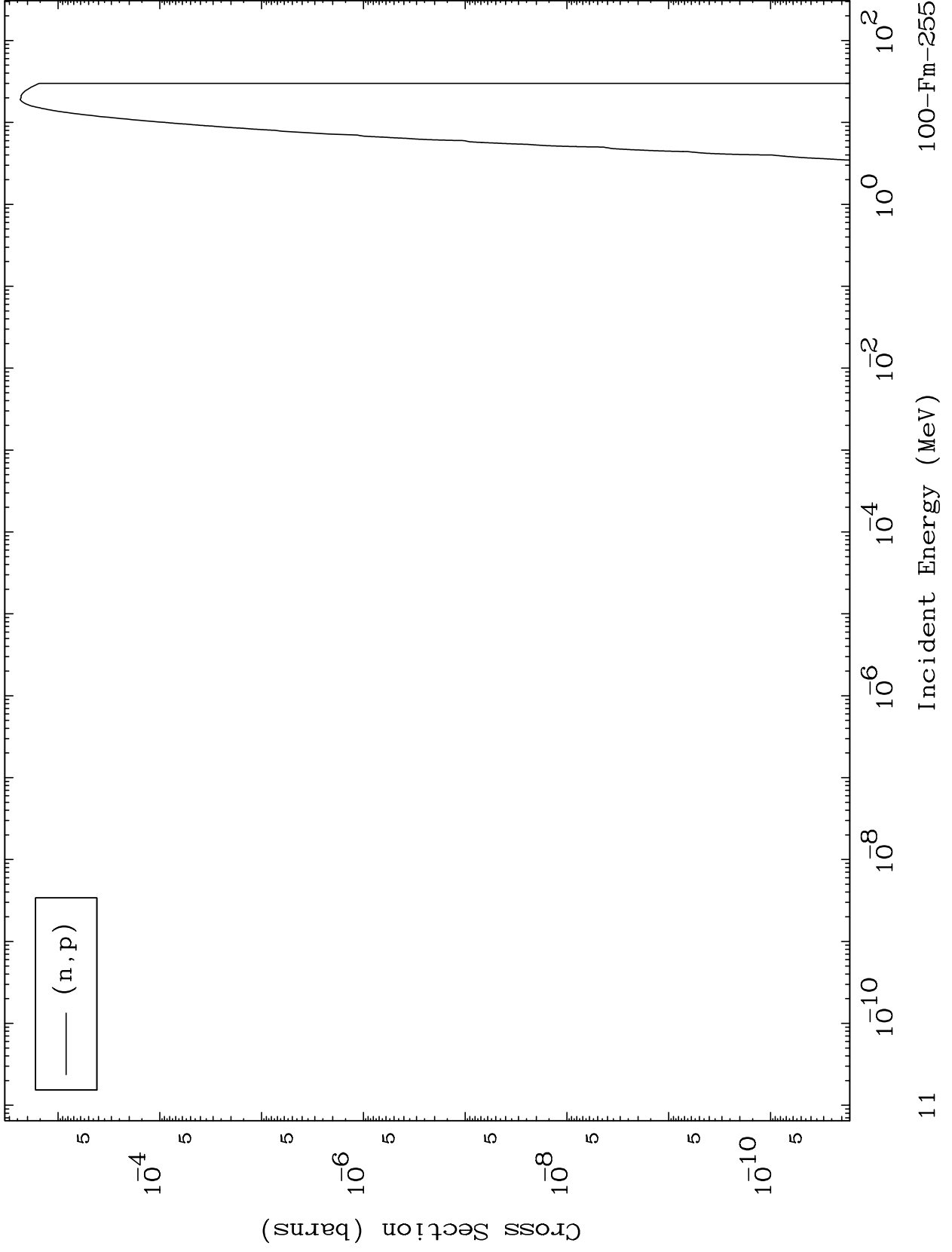
100-Fm-255



MAT 9936

(n,p) Levels
293 Kelvin Cross Sections

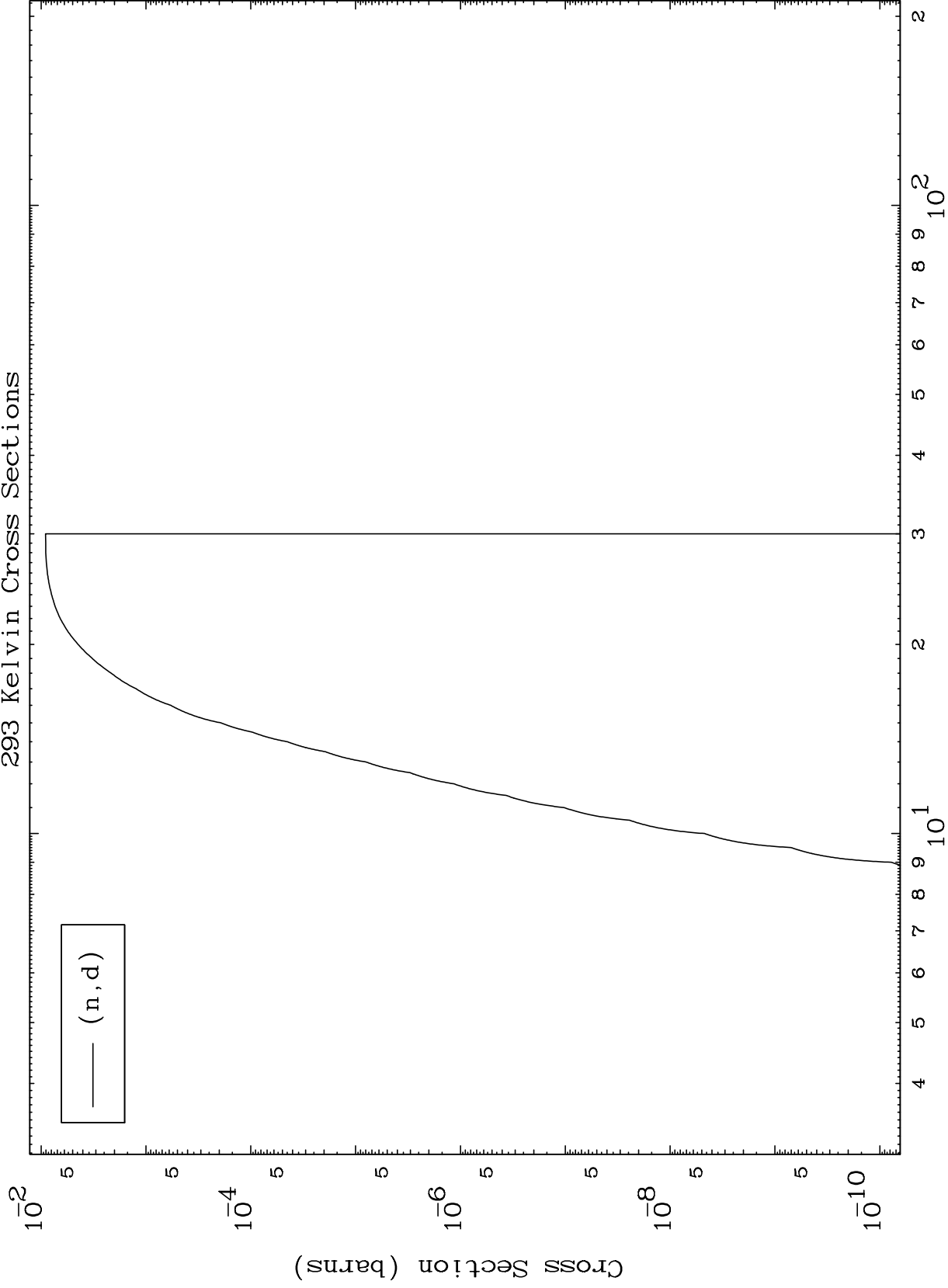
100-Fm-255



MAT 9936

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-255



12

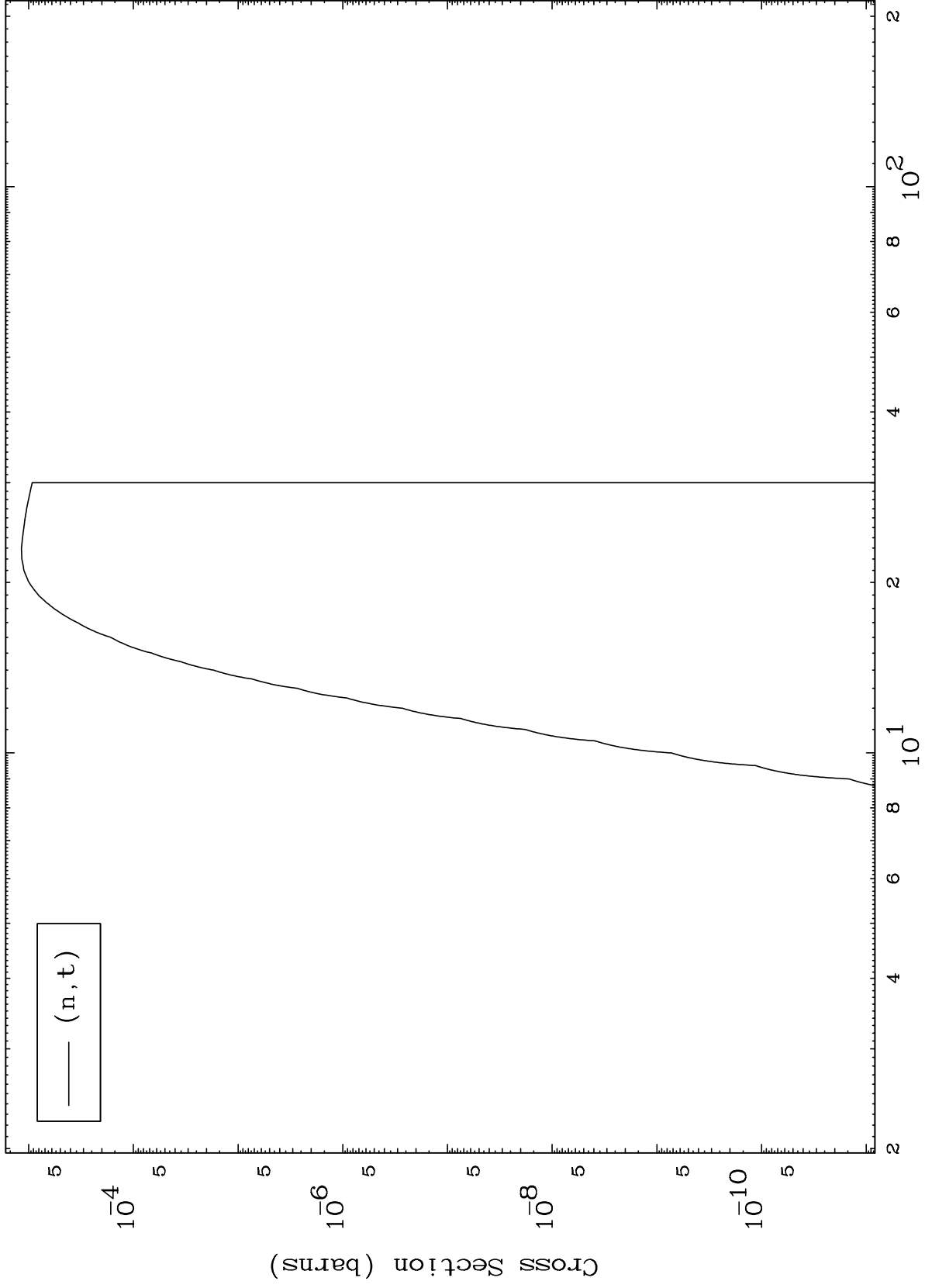
Incident Energy (MeV)

100-Fm-255

MAT 9936

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-255



13

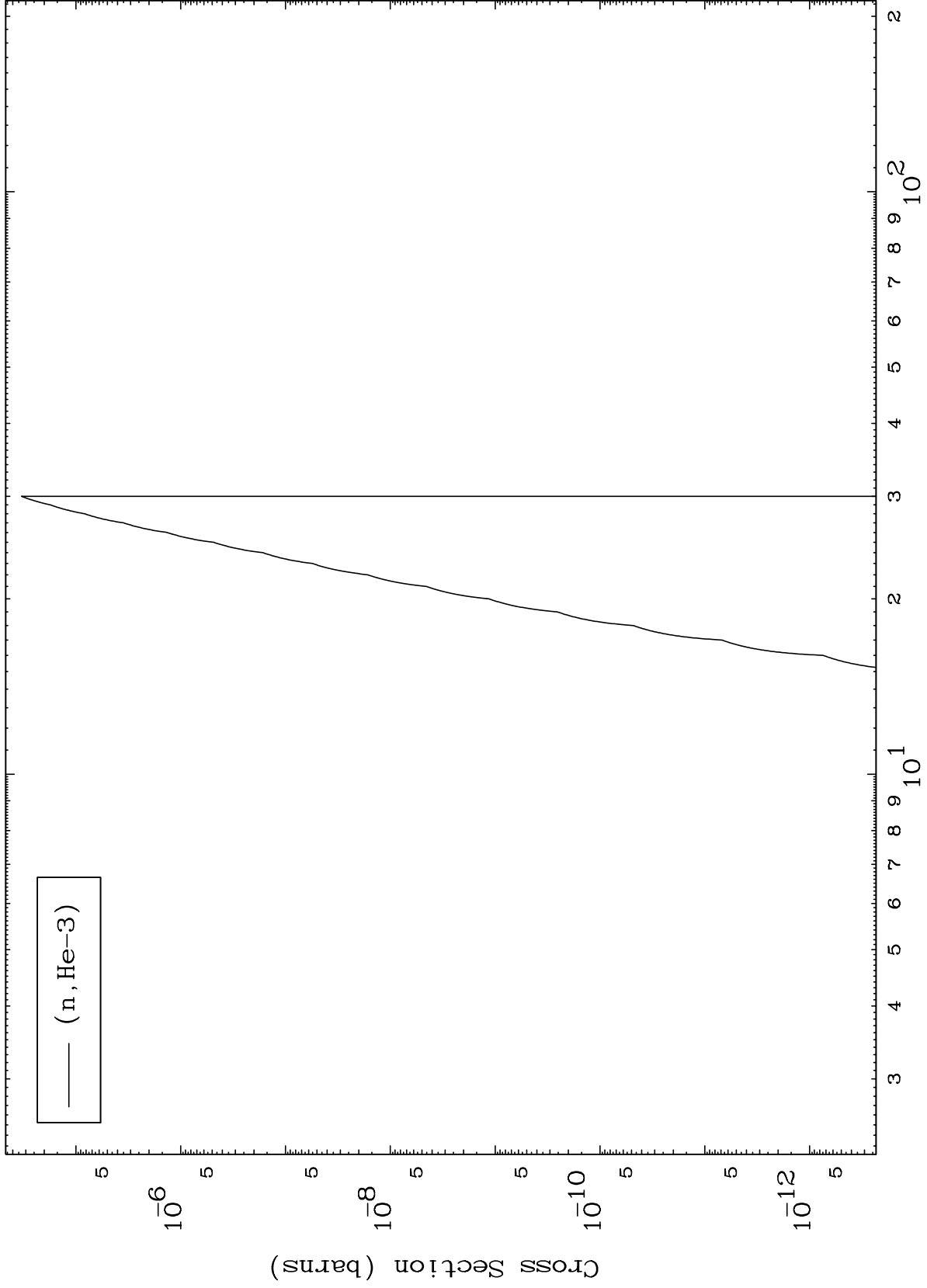
Incident Energy (MeV)

100-Fm-255

MAT 9936

(n,He3) Levels
293 Kelvin Cross Sections

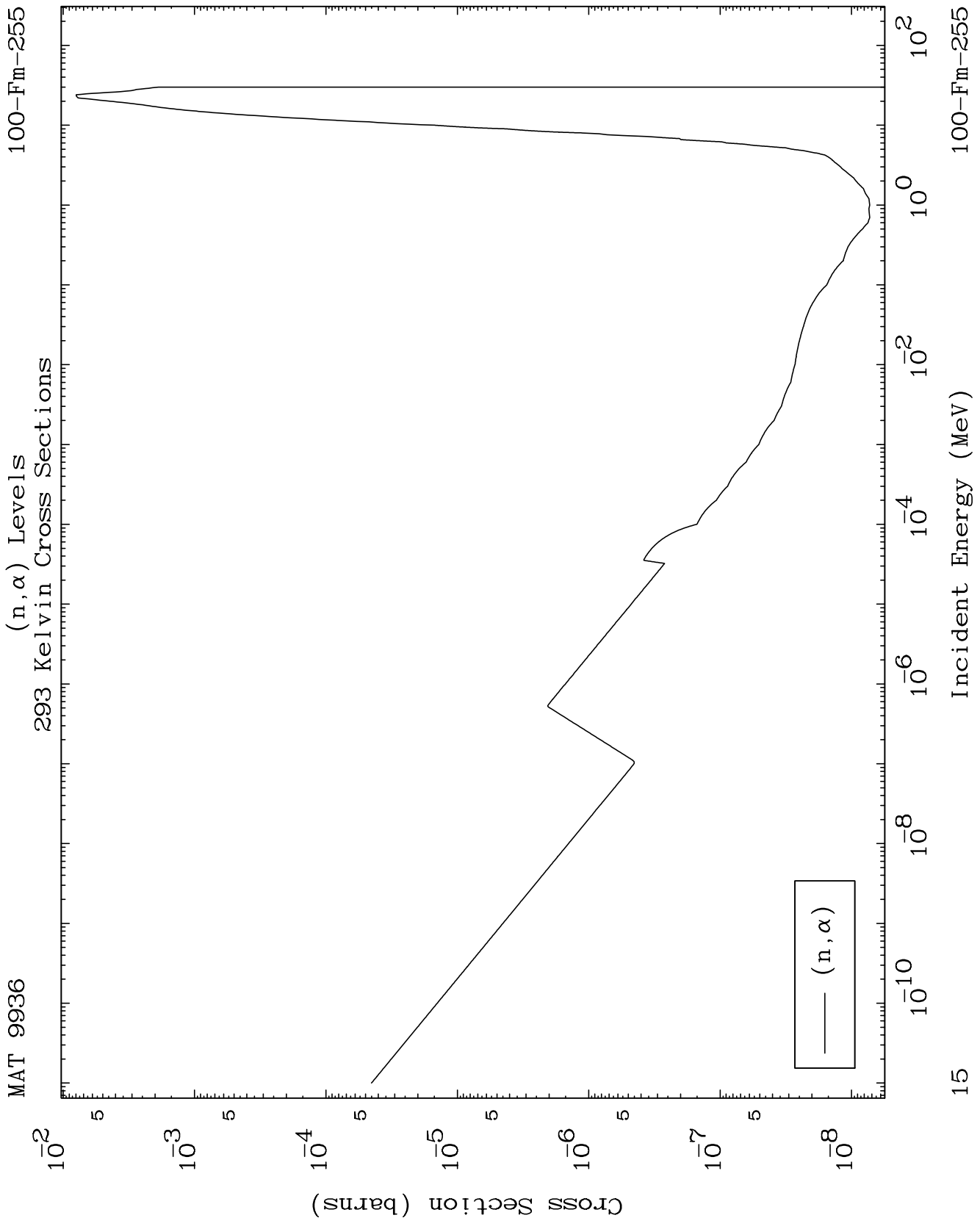
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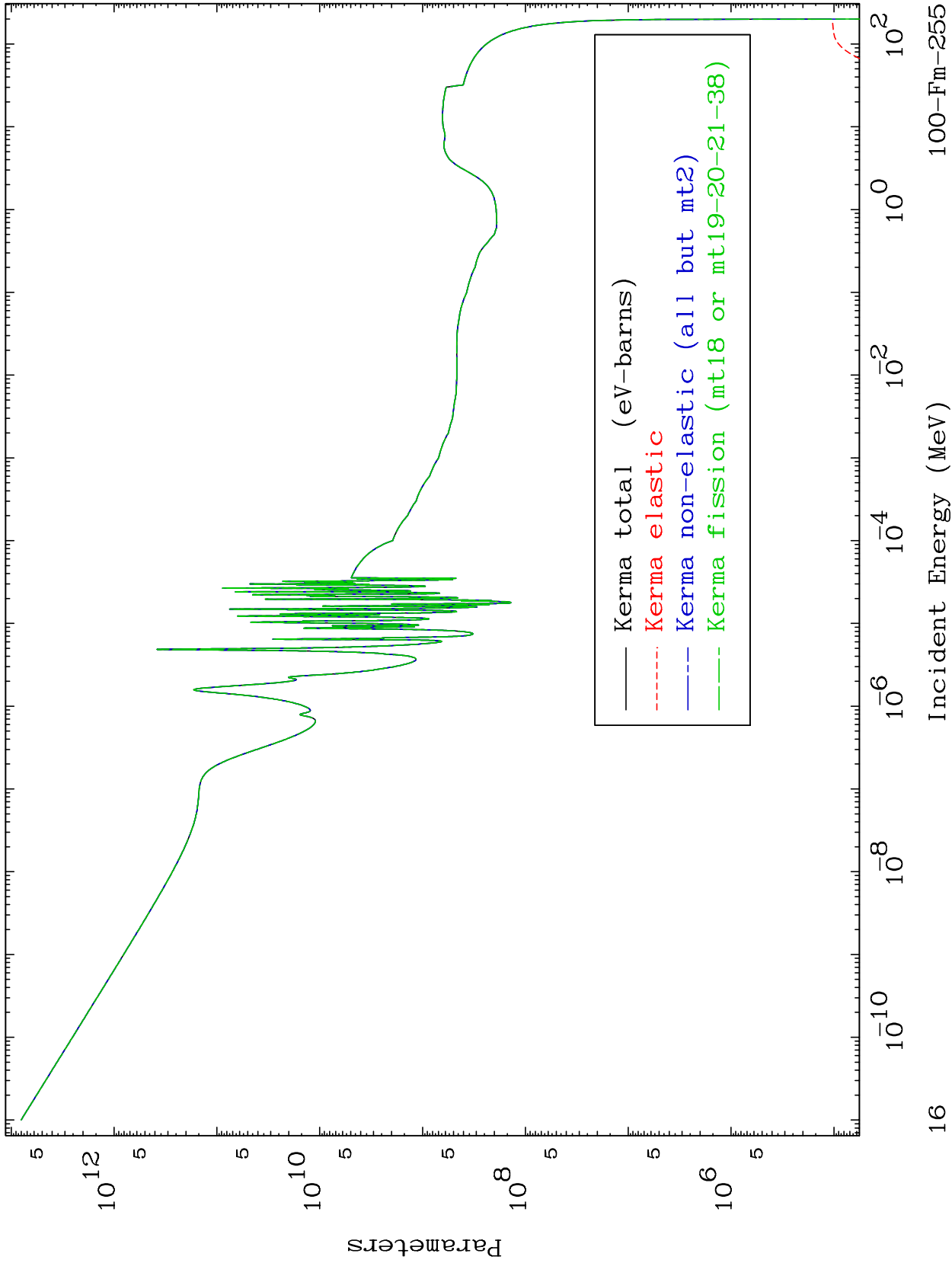


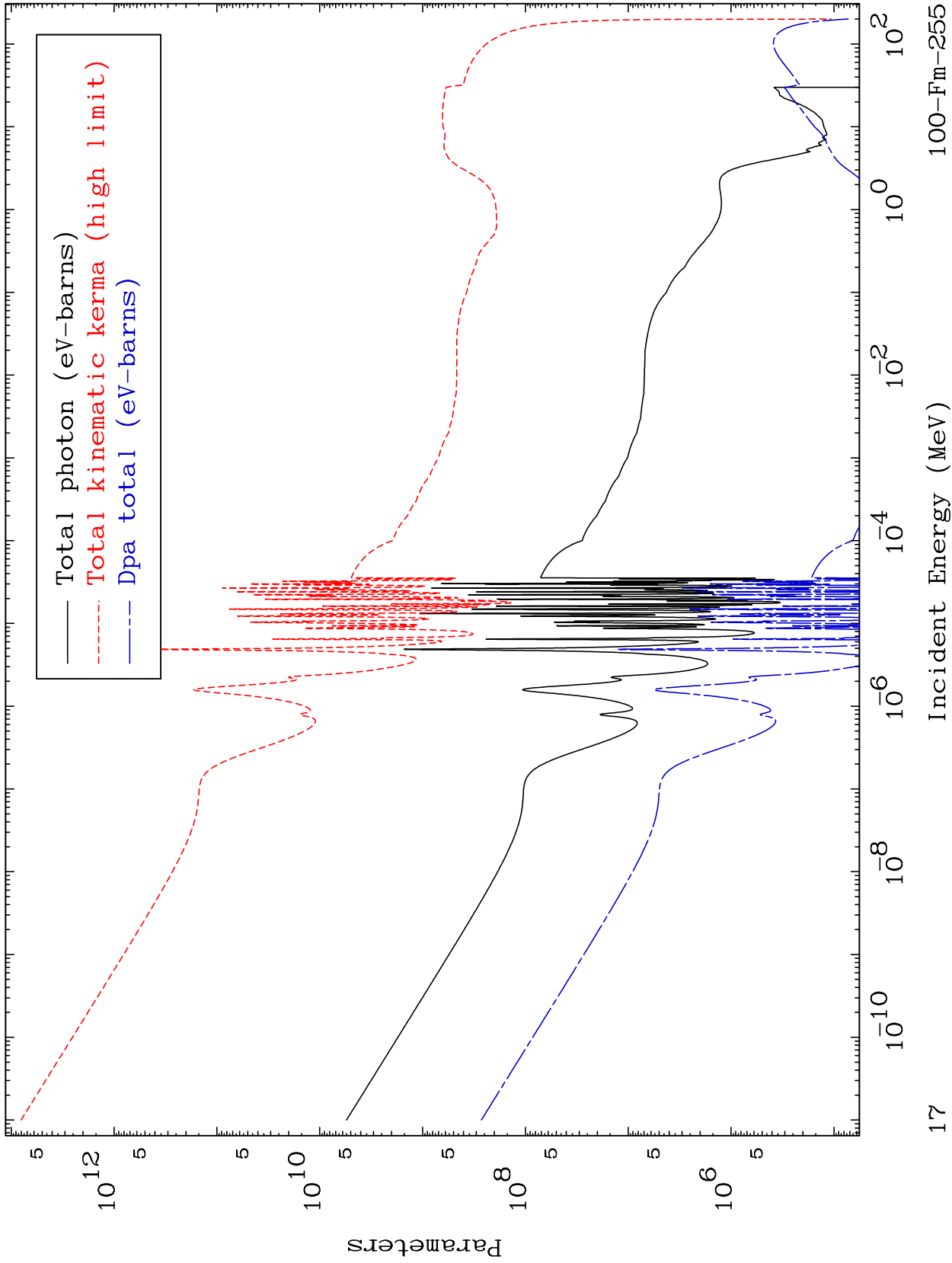
14

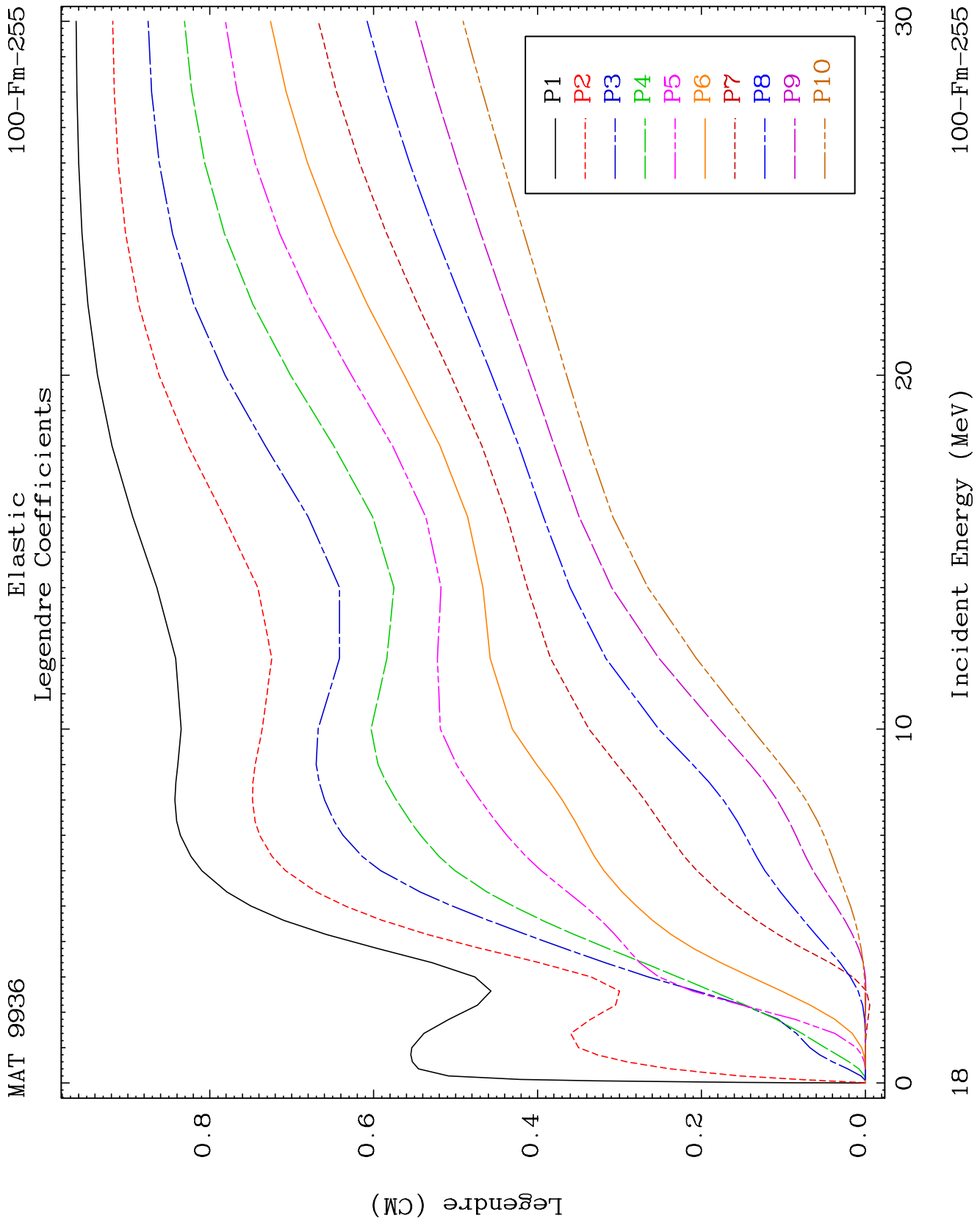
Incident Energy (MeV)

100-Fm-255





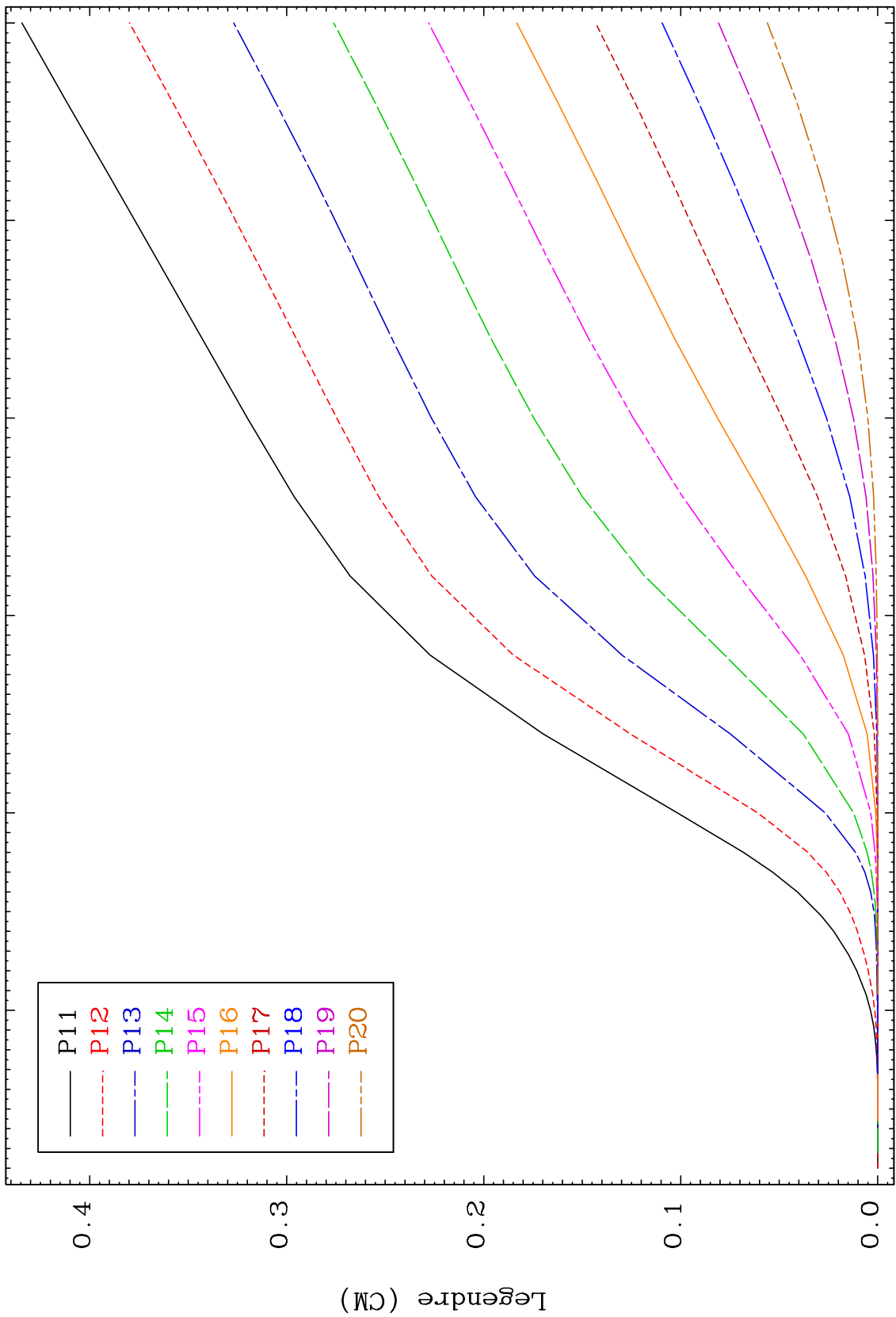




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Elastic Legendre Coefficients

100-Fm-255



19

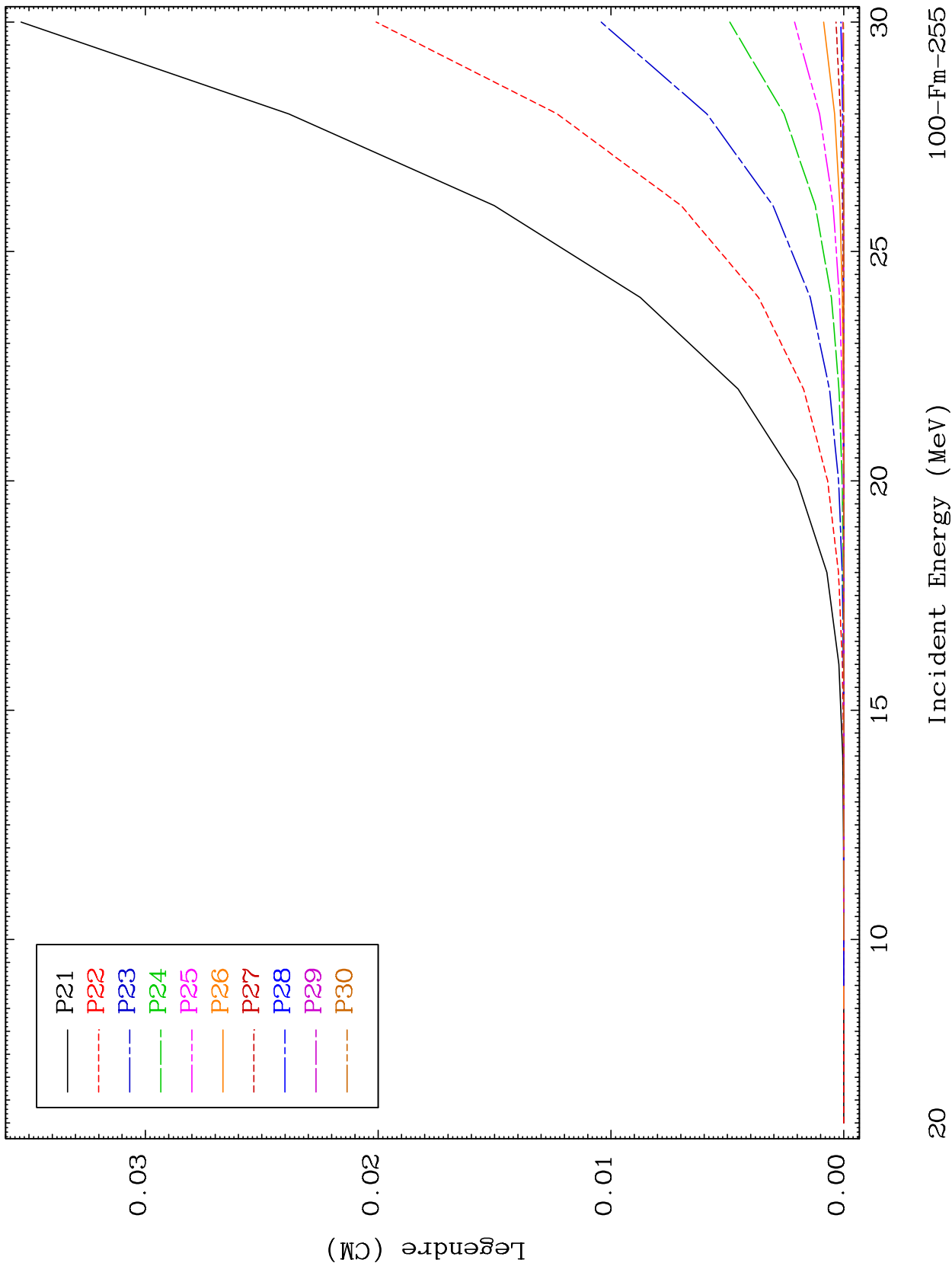
Incident Energy (MeV)

100-Fm-255

MAT 9936

Elastic
Legendre Coefficients

100-Fm-255



20

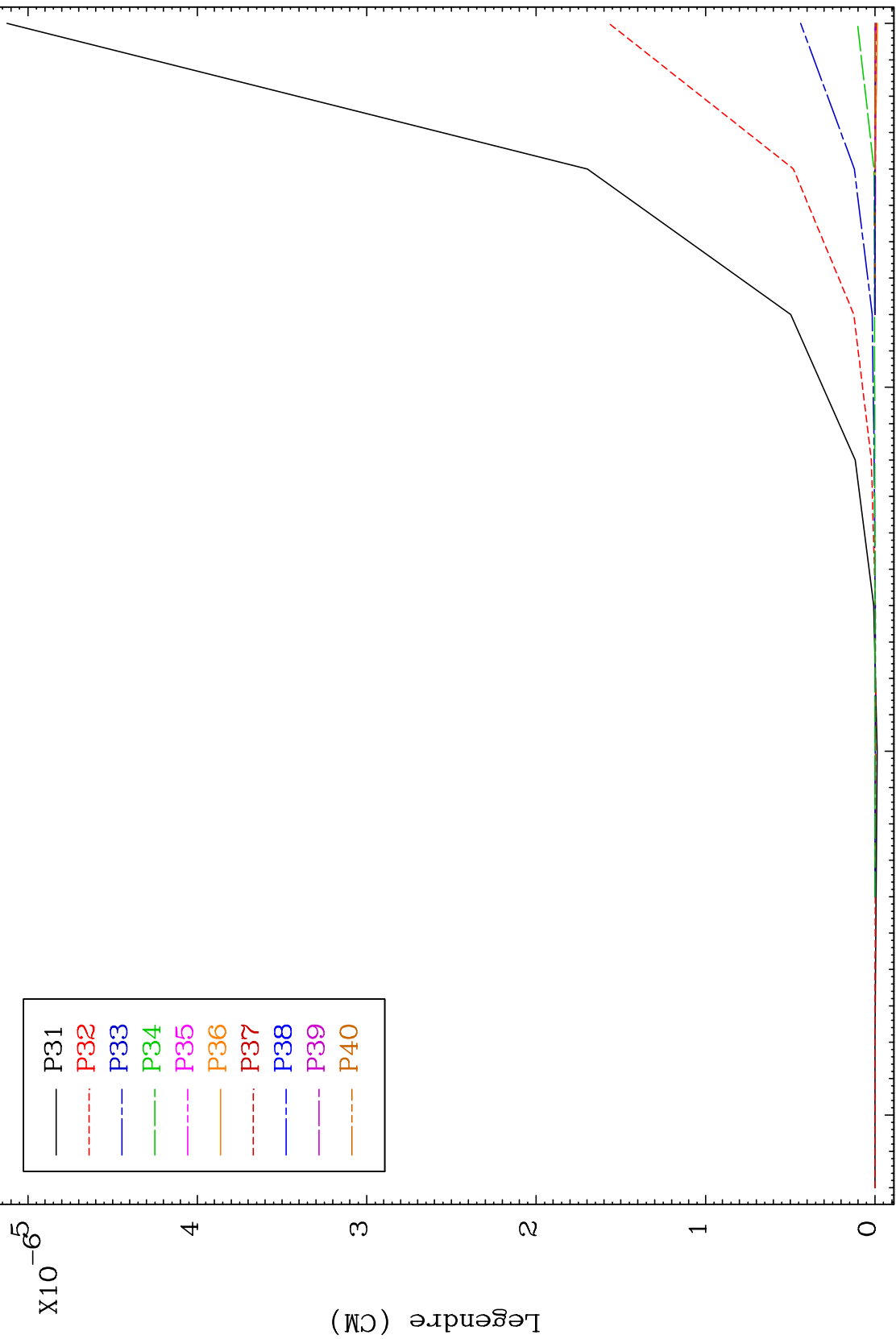
Incident Energy (MeV)

100-Fm-255

MAT 9936

Elastic
Legendre Coefficients

100-Fm-255



21

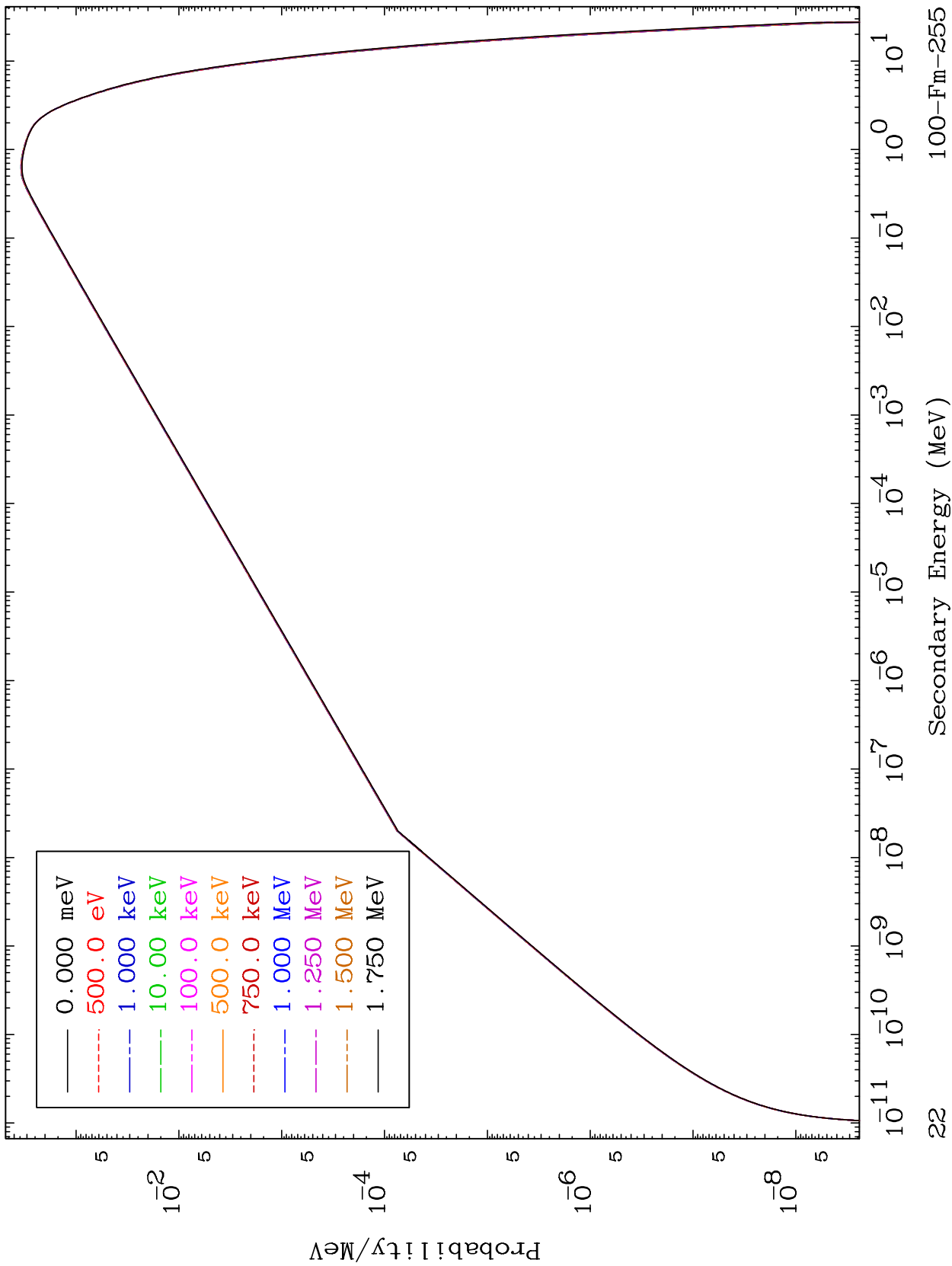
Incident Energy (MeV)

100-Fm-255

MAT 9936

Fission Energy Distribution

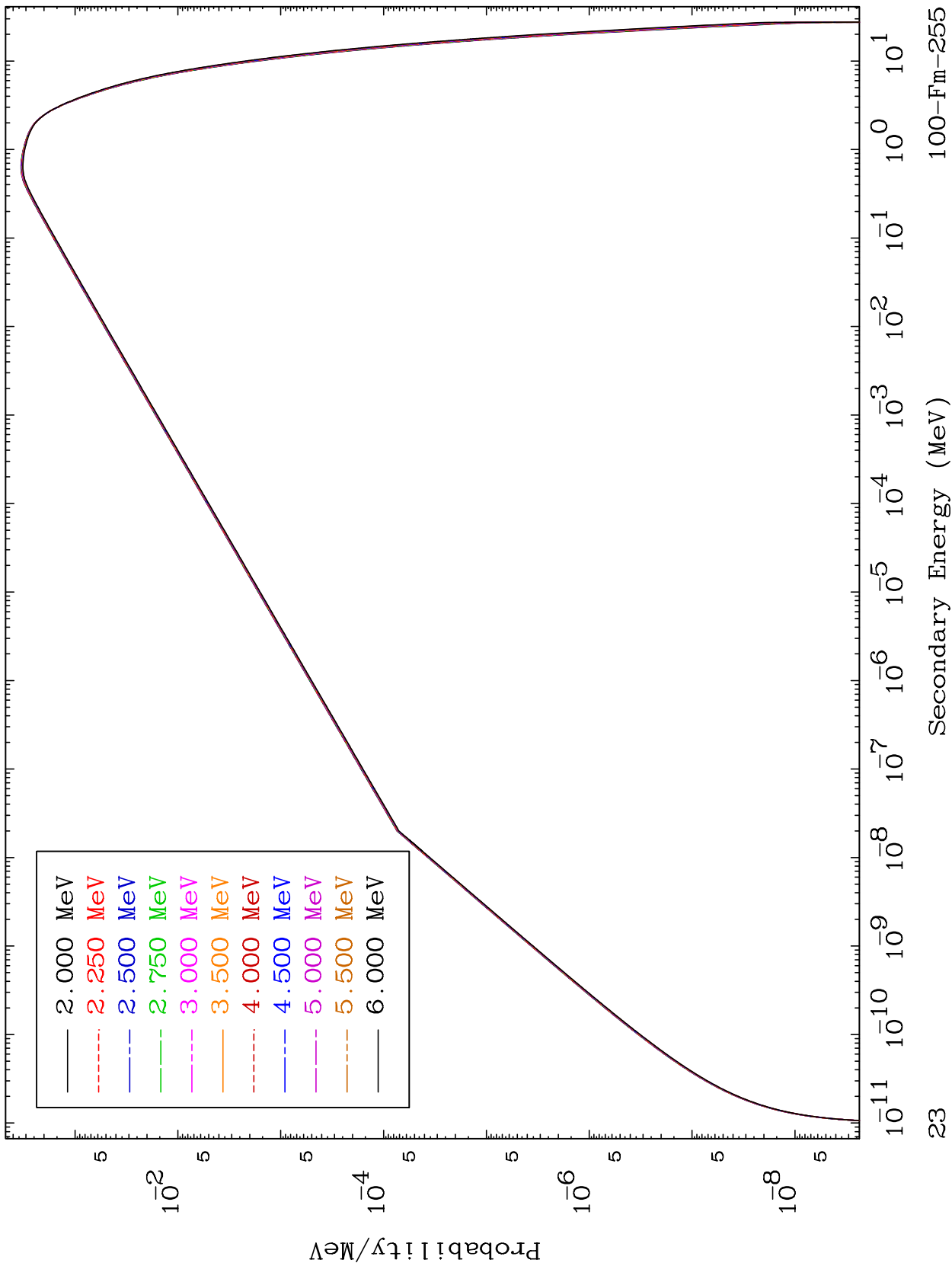
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Fission Energy Distribution

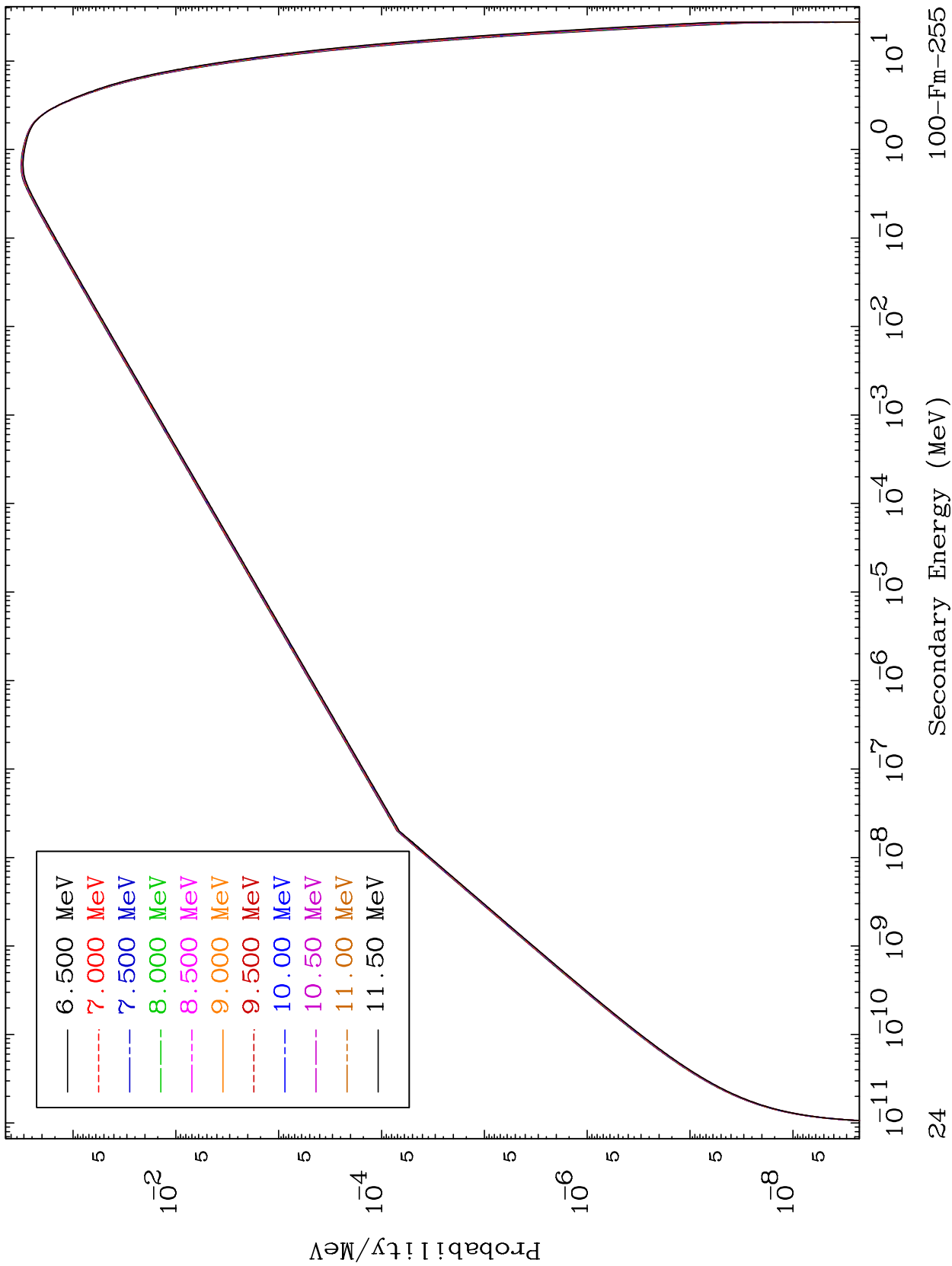
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MAT 9936

Fission Energy Distribution

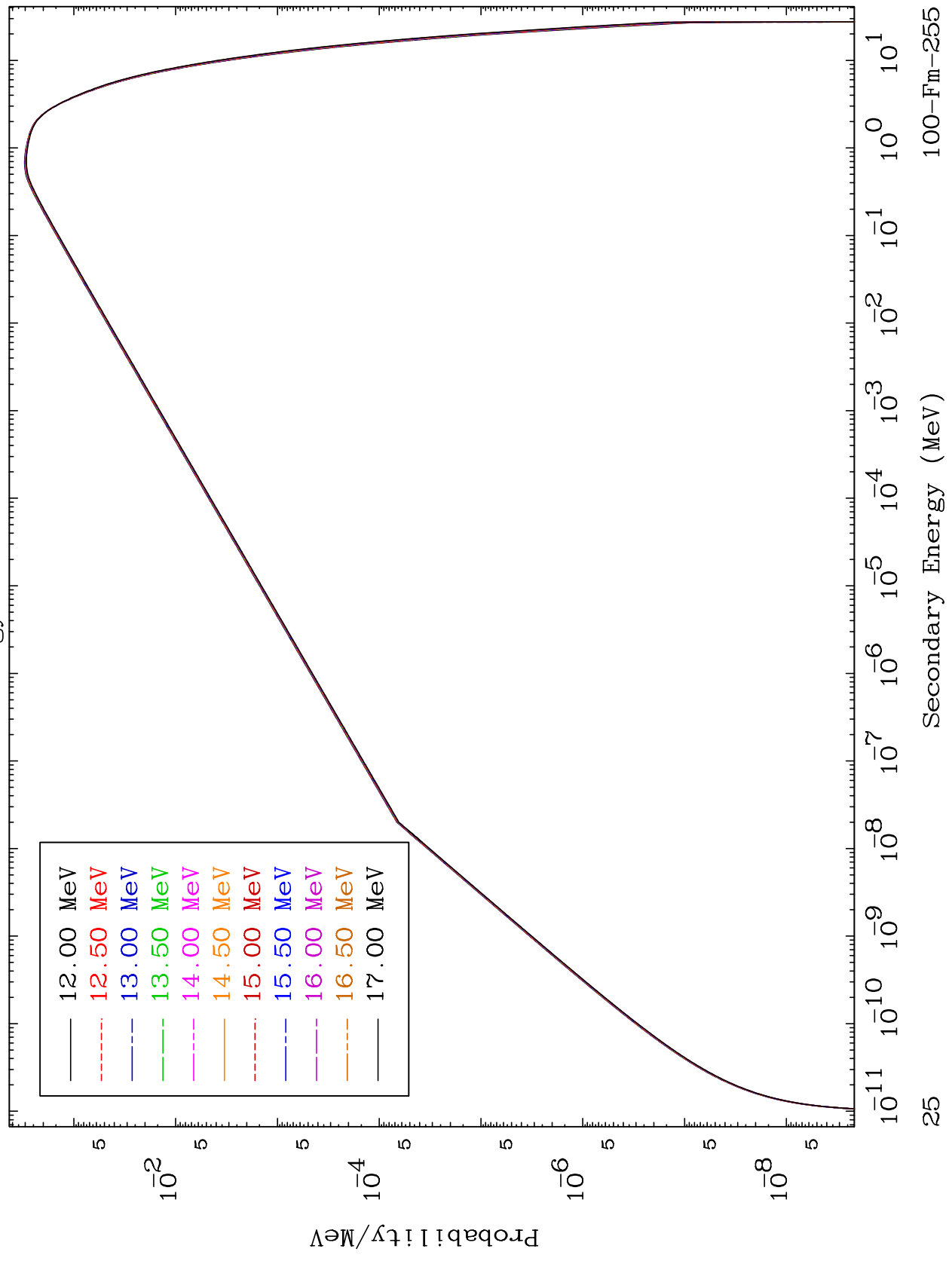
100-Fm-255



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Fission Energy Distribution

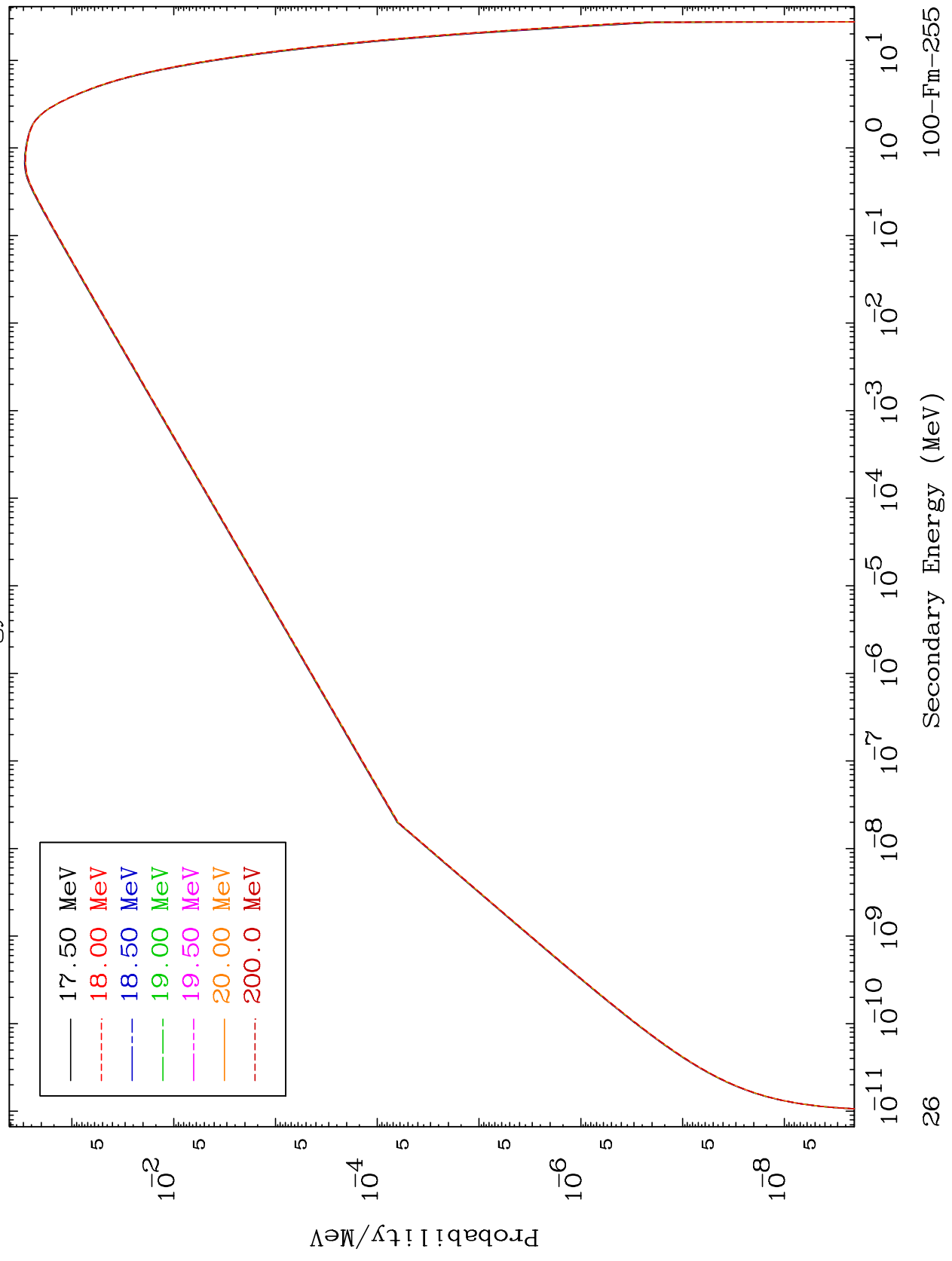
100-Fm-255



MAT 9936

Fission Energy Distribution

100-Fm-255

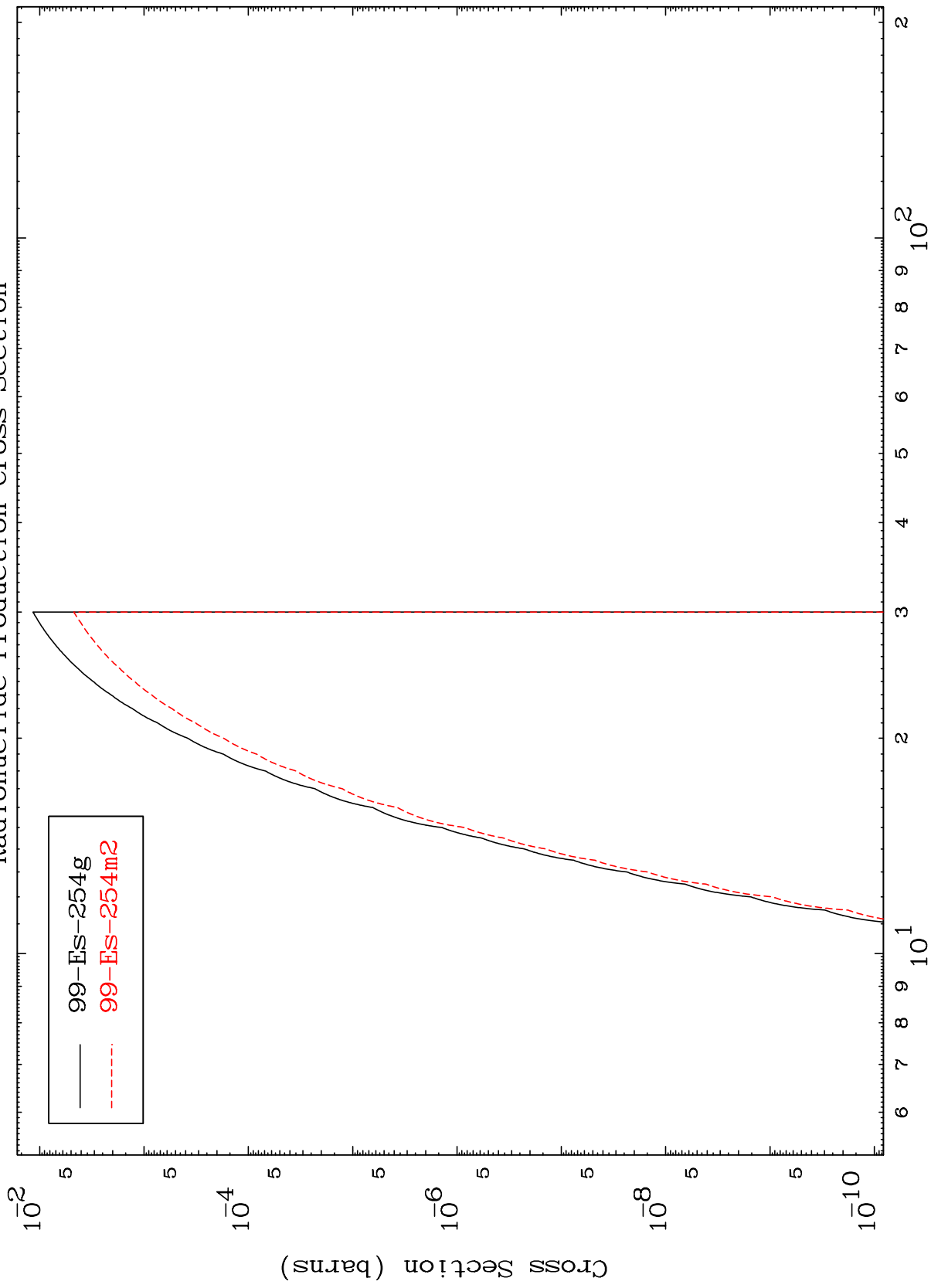


MAT 9936

(n,n') p

100-Fm-255

Radionuclide Production Cross Section



27

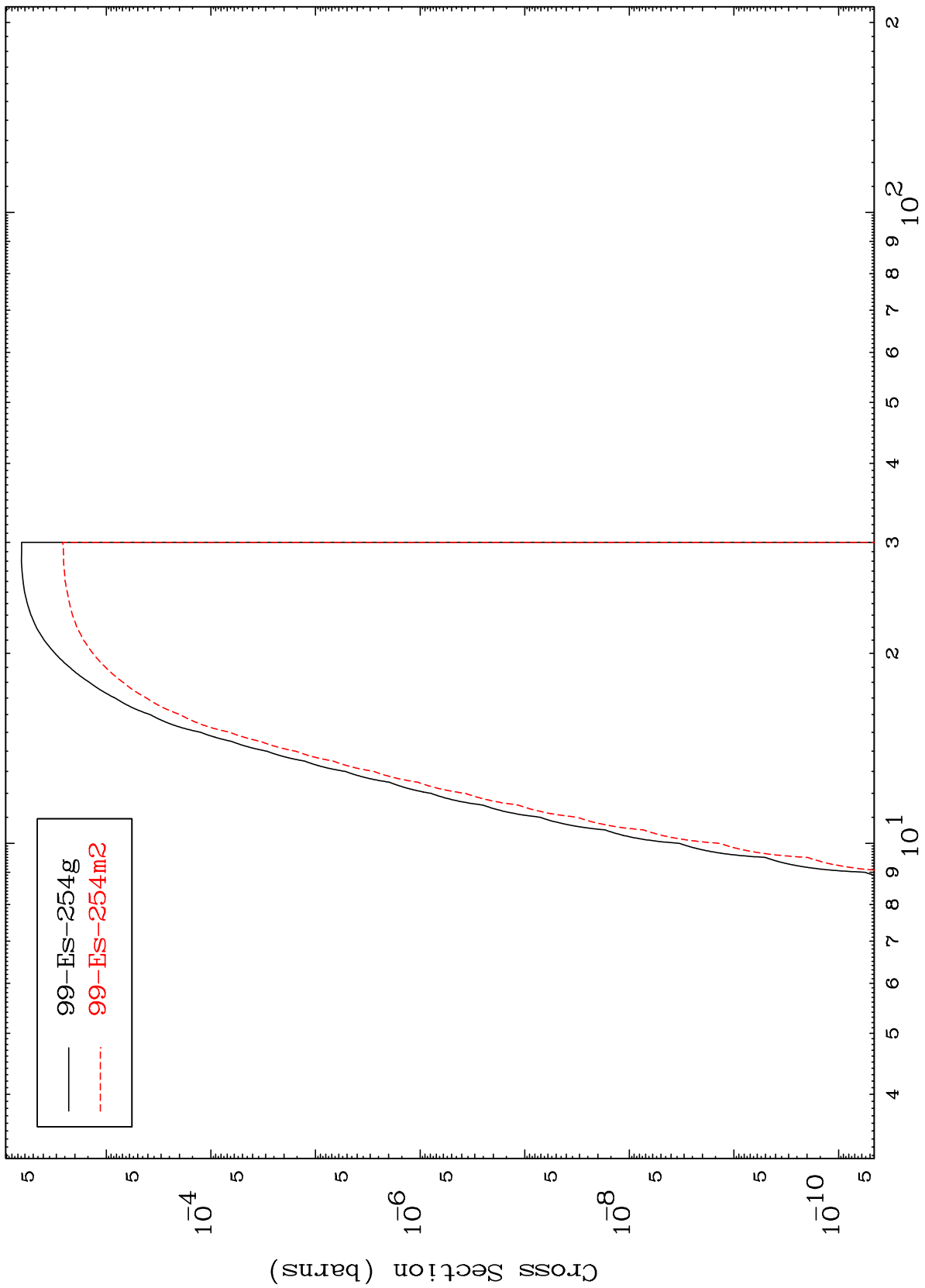
Incident Energy (MeV)

100-Fm-255

MAT 9936

100-Fm-255

(n,d)
Radionuclide Production Cross Section



28

Incident Energy (MeV)

100-Fm-255