

Program EVALPLOT  
(Version 2018-1)

by

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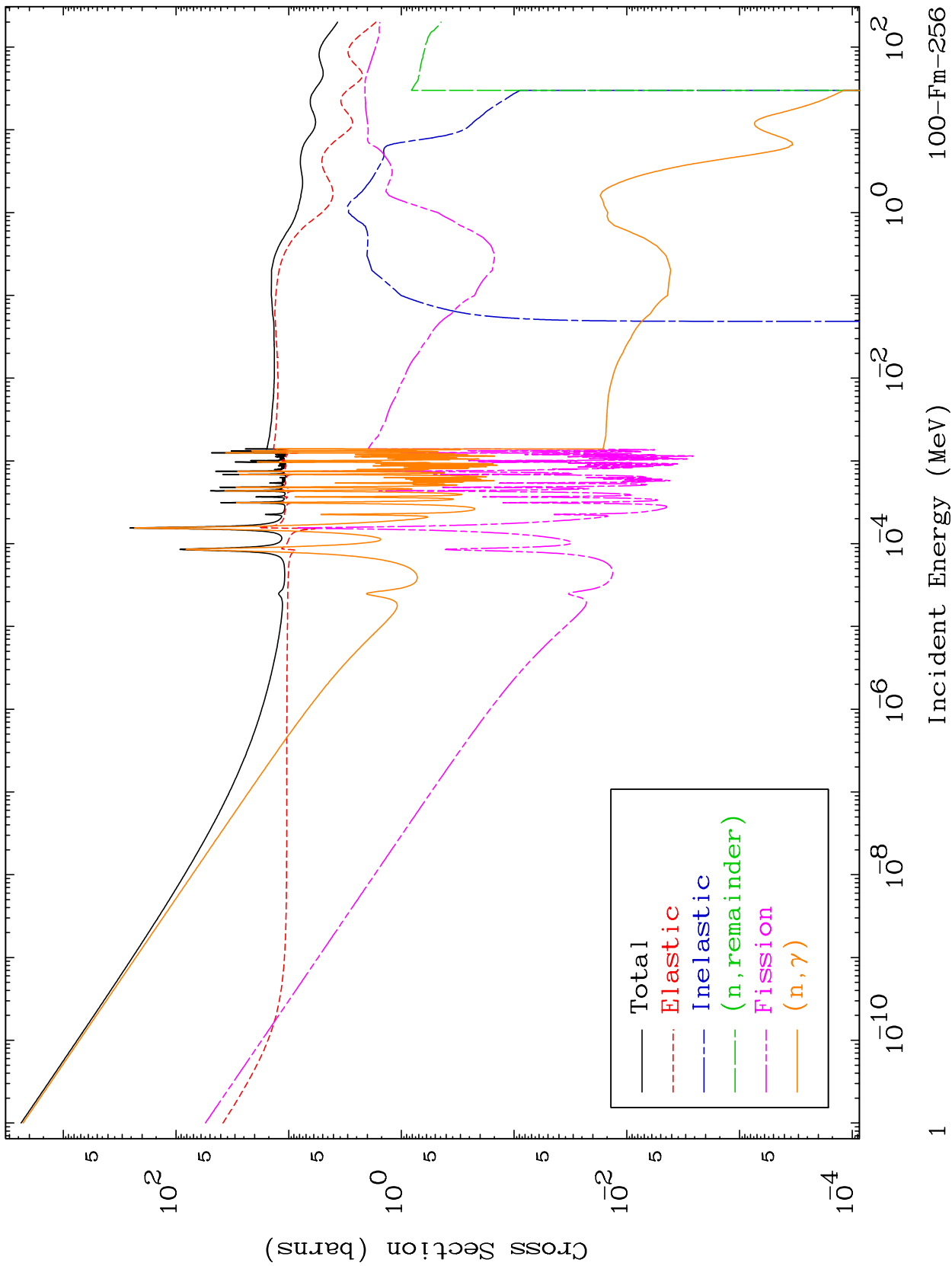
Press Mouse Button to Start

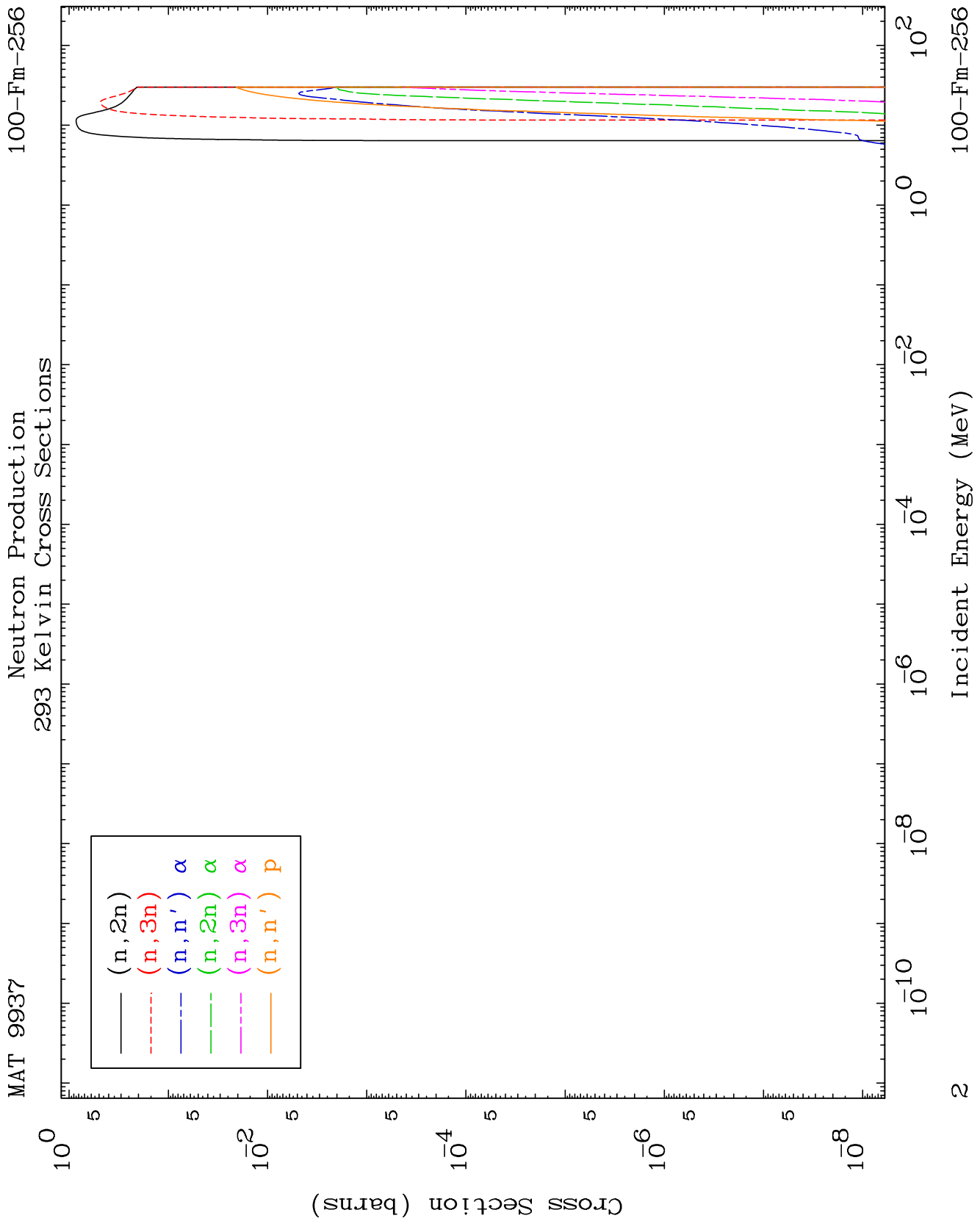
MAT 9937

Major

293 Kelvin Cross Sections

100-Fm-256

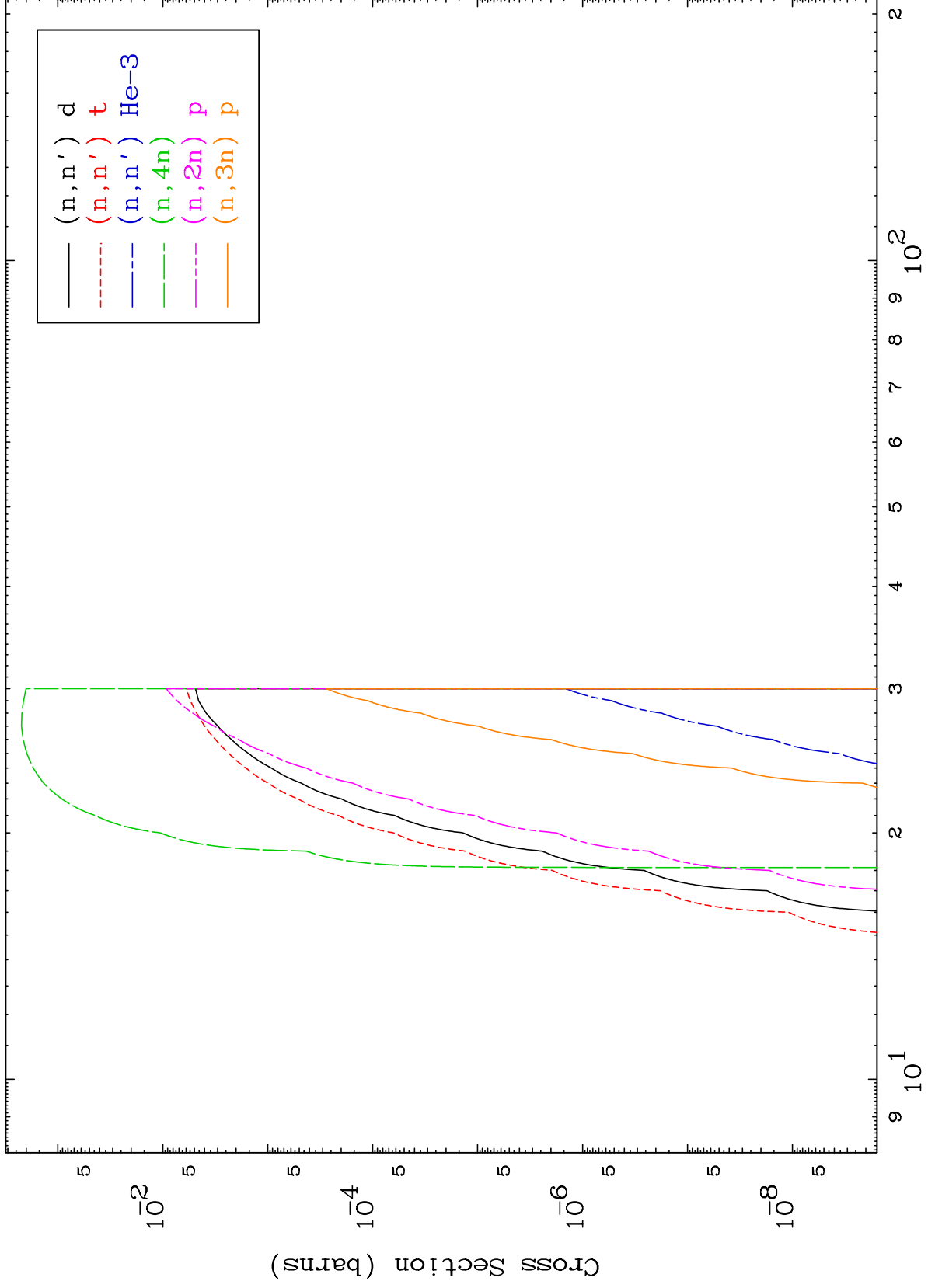




MAT 9937

Neutron Production  
293 Kelvin Cross Sections

100-Fm-256



3

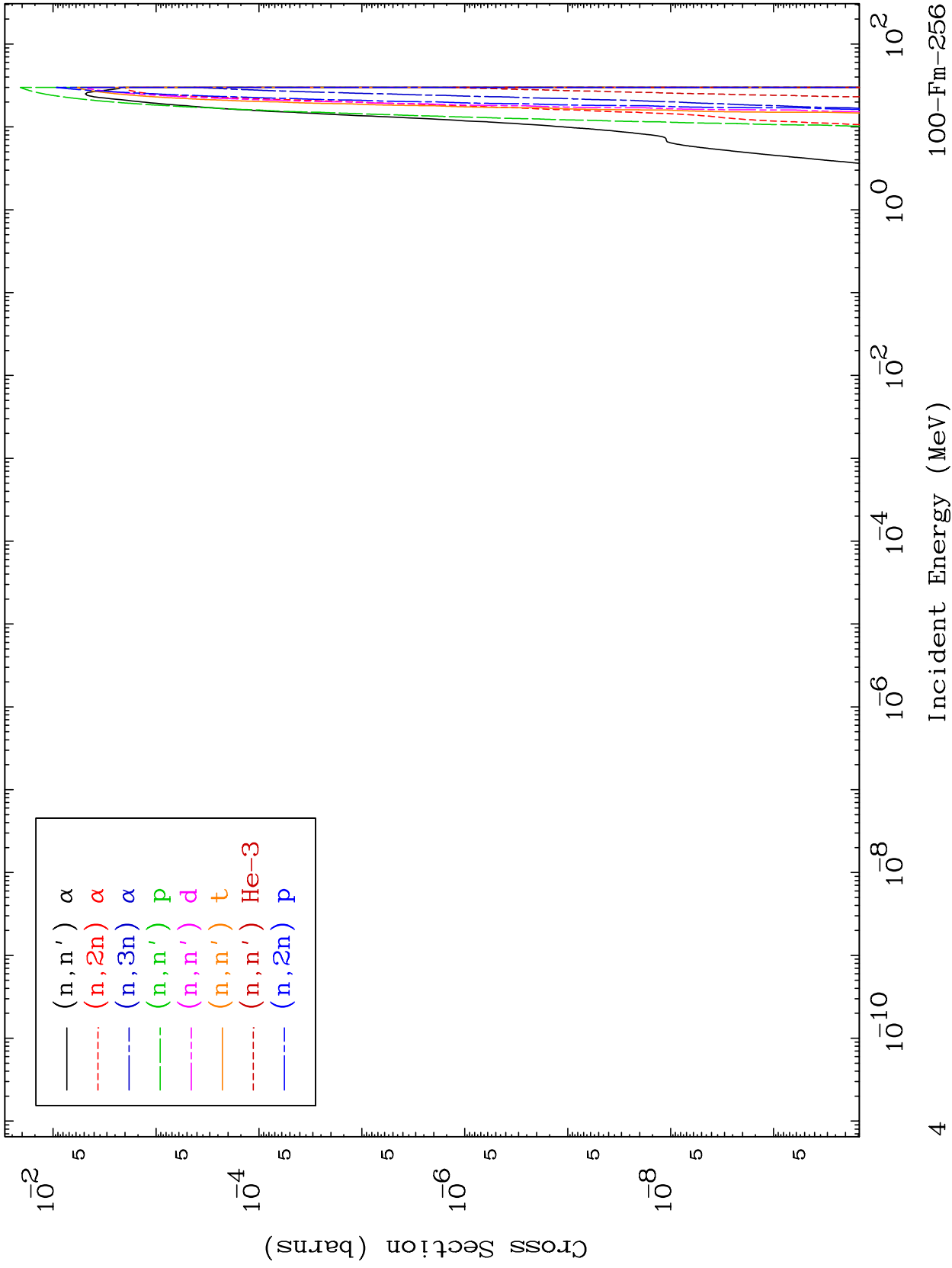
Incident Energy (MeV)

100-Fm-256

MAT 9937

Charged Particle  
293 Kelvin Cross Sections

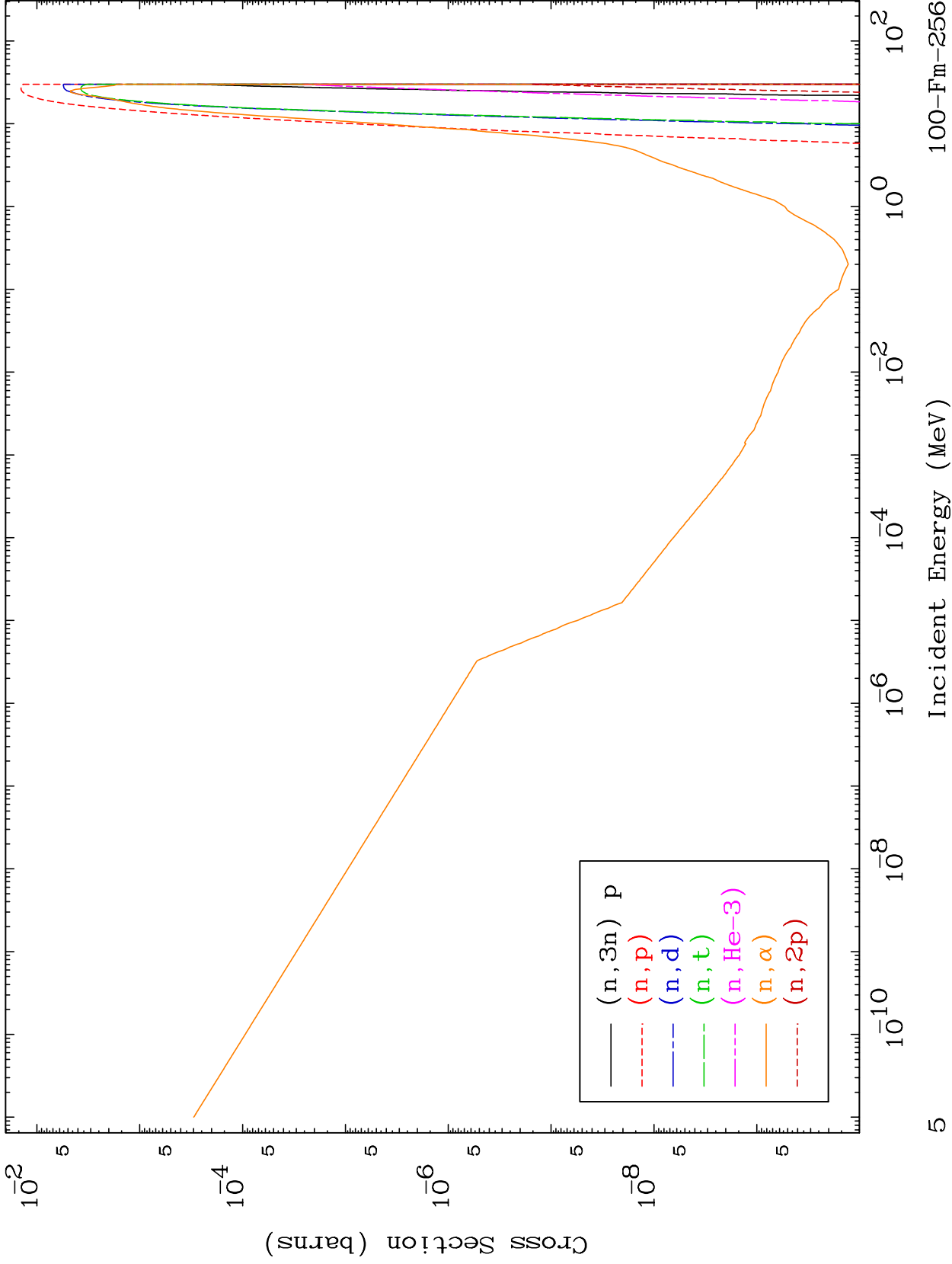
100-Fm-256



MAT 9937

Charged Particle  
293 Kelvin Cross Sections

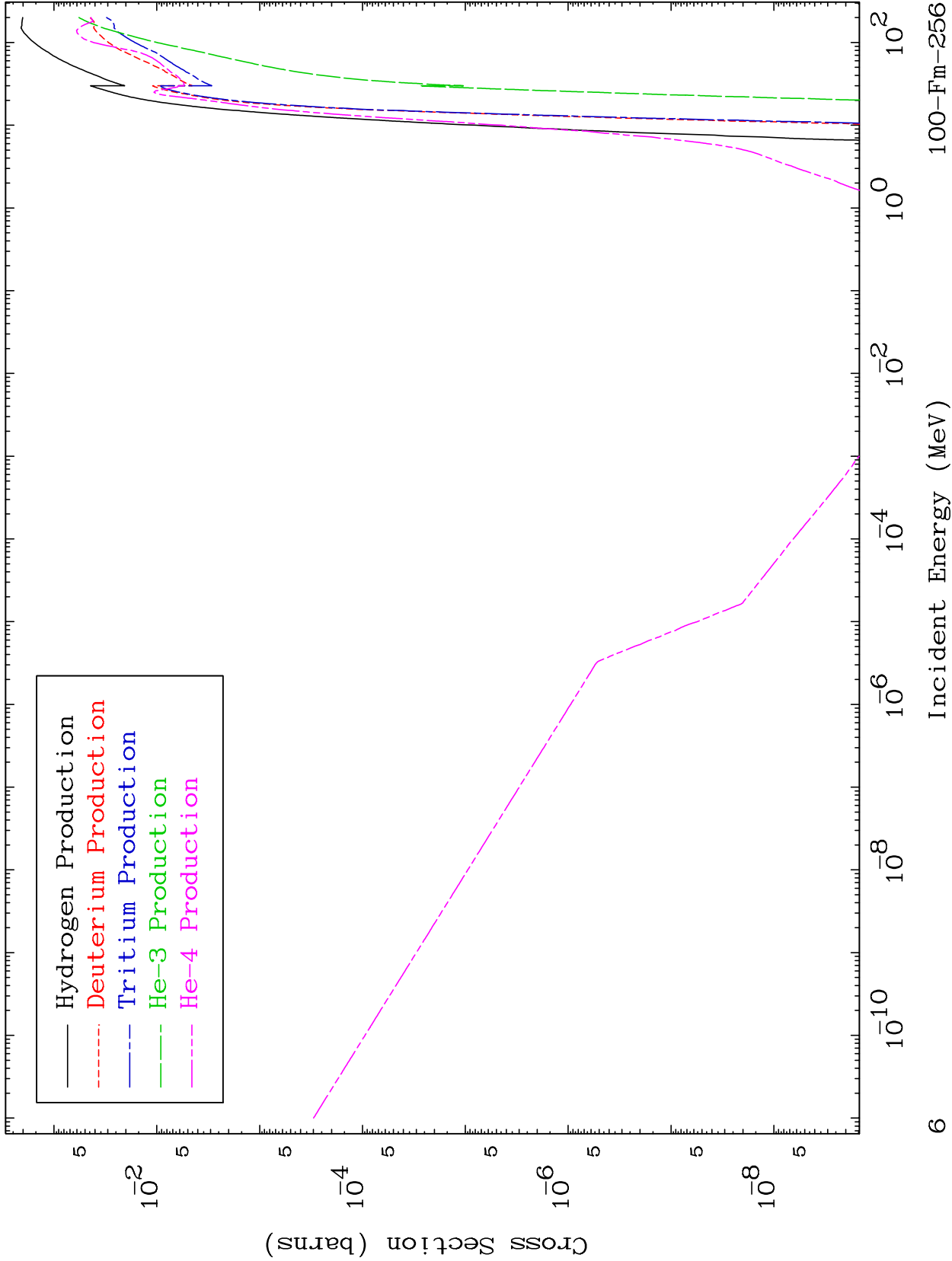
100-Fm-256



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Particle Production  
293 Kelvin Cross Sections

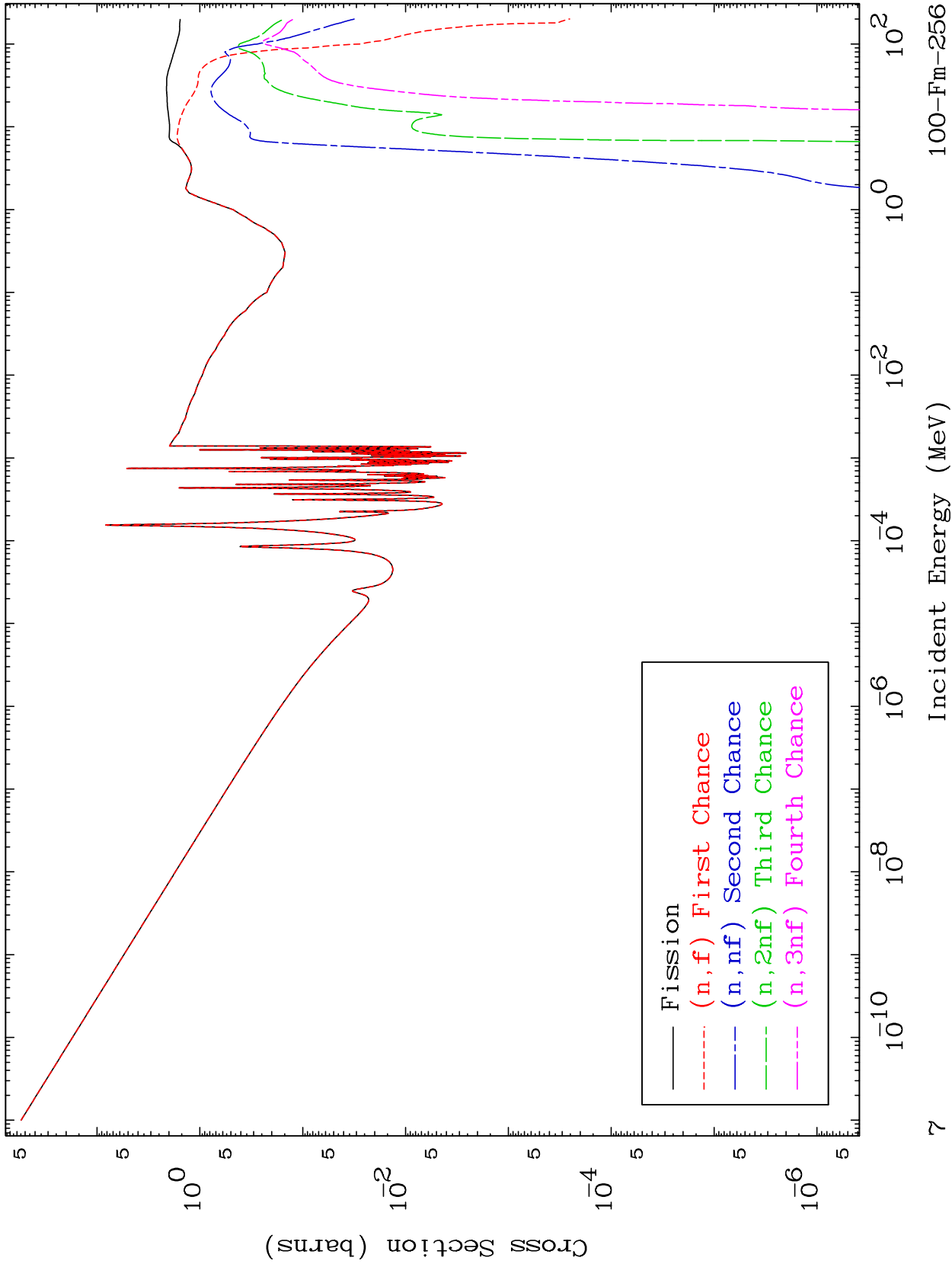
100-Fm-256



MAT 9937

Fission  
293 Kelvin Cross Sections

100-Fm-256



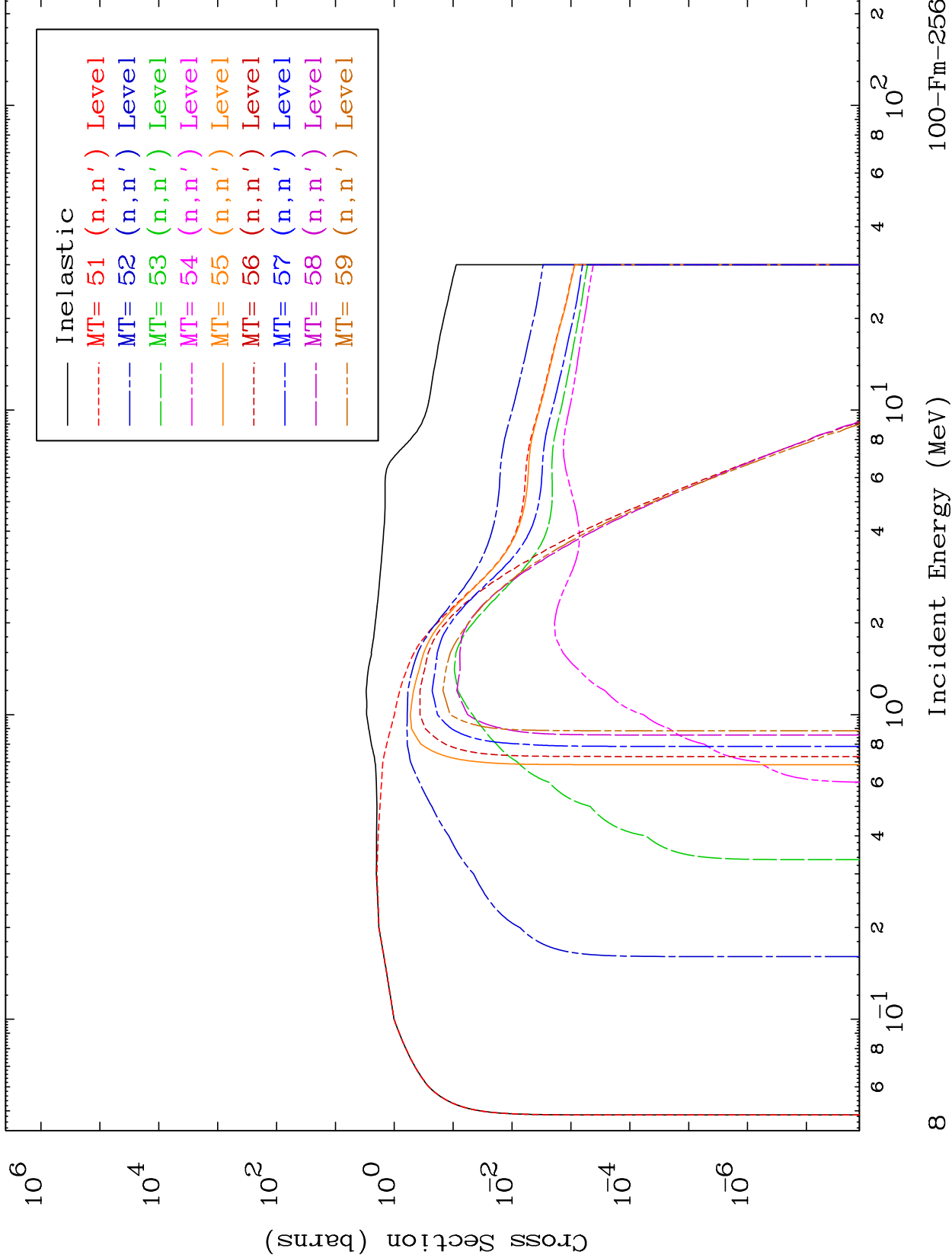


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(n,n') Level

100-Fm-256

293 Kelvin Cross Sections

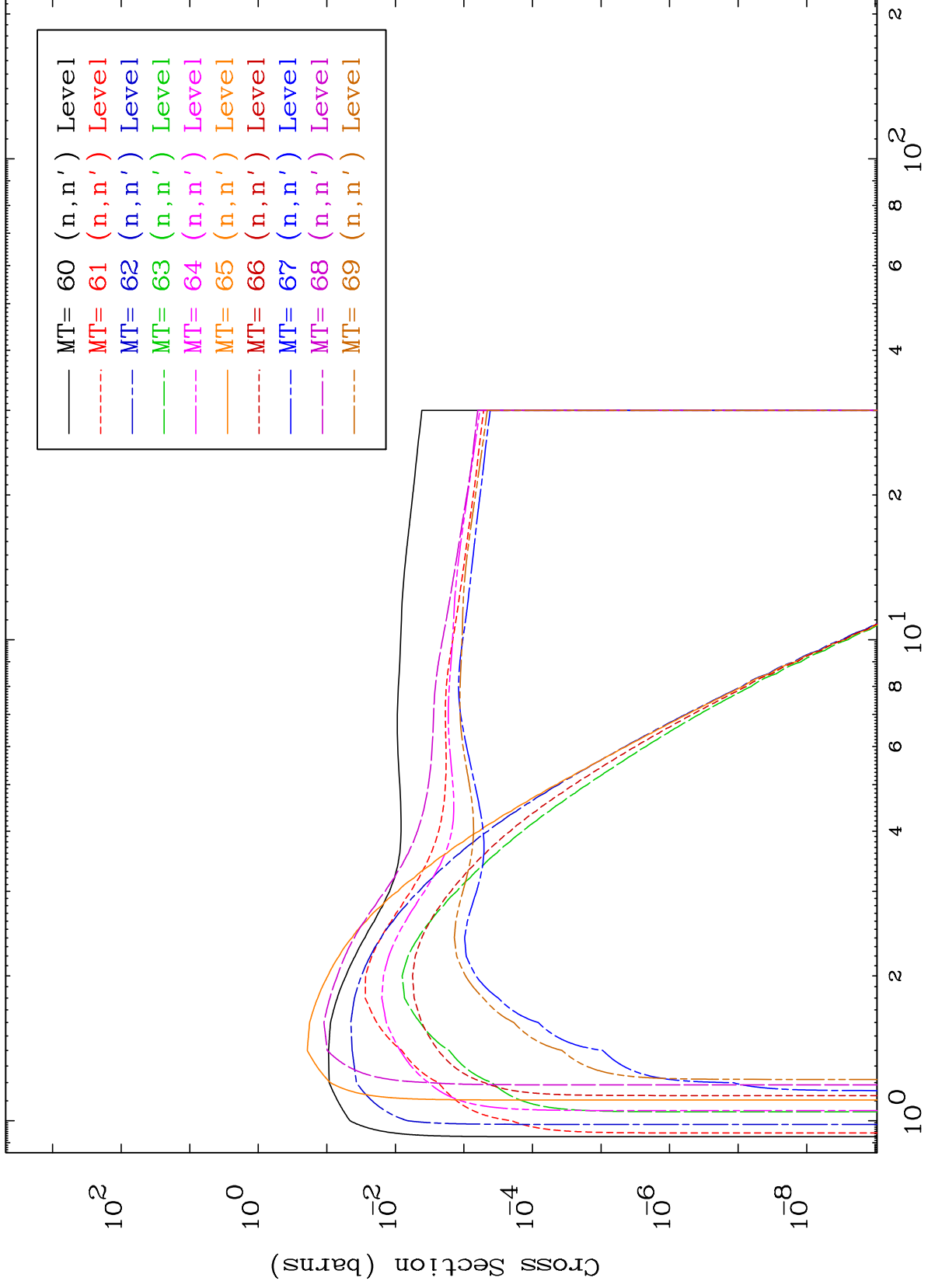


MAT 9937

(n,n') Level

100-Fm-256

293 Kelvin Cross Sections



Incident Energy (MeV)

100-Fm-256

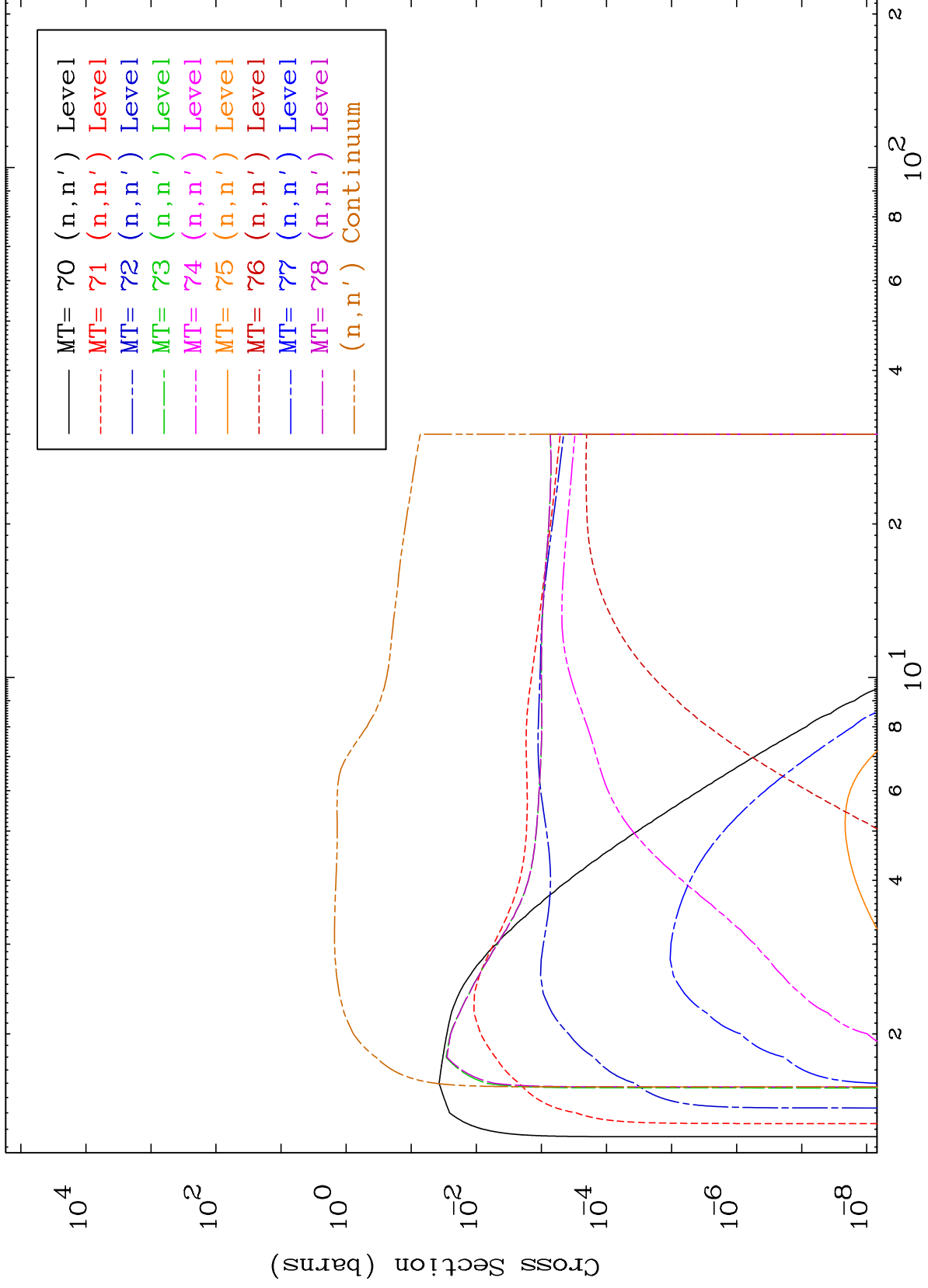
9

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(n,n') Level

293 Kelvin Cross Sections

100-Fm-256



10

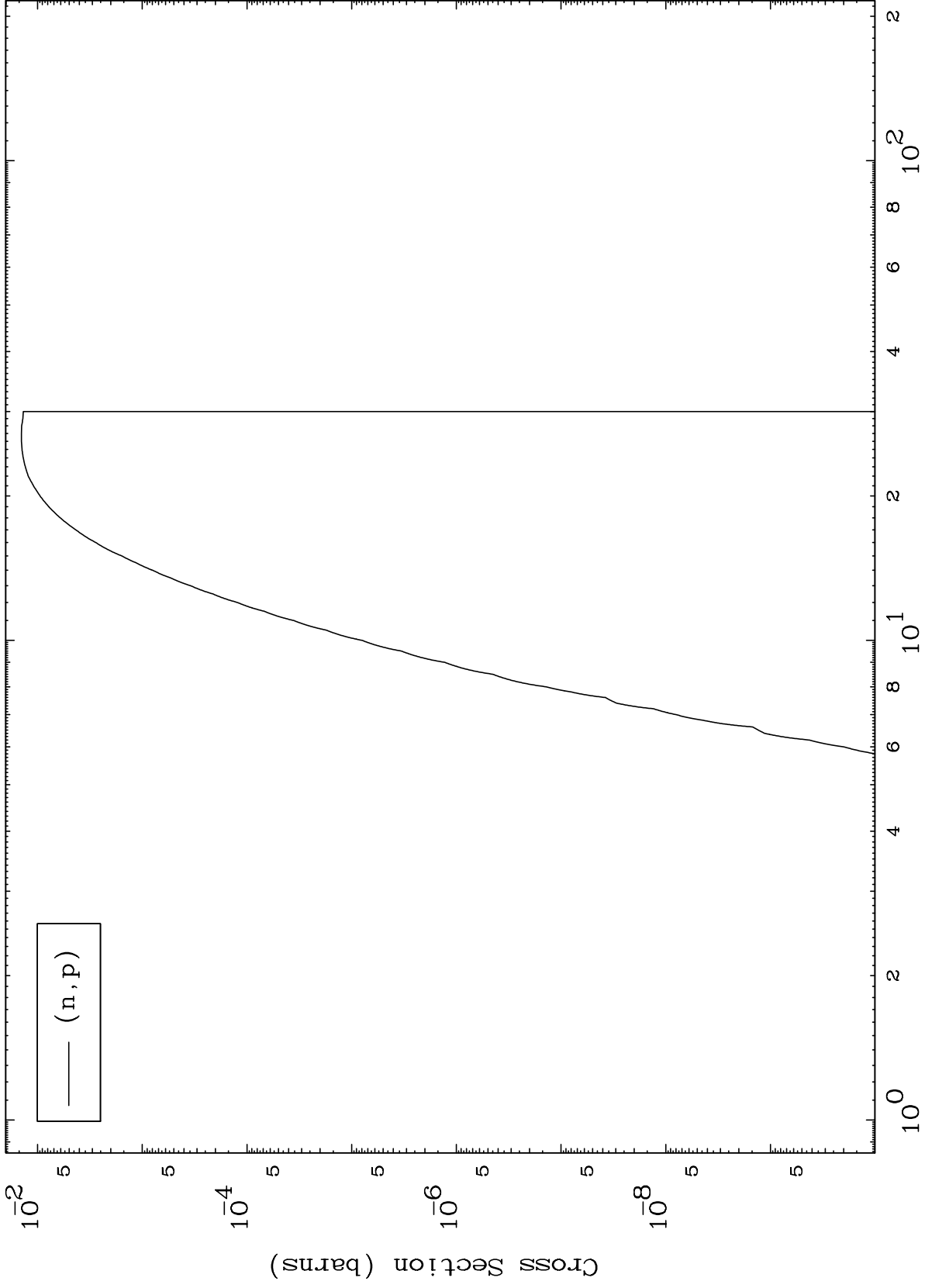
Incident Energy (MeV)

100-Fm-256

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(n,p) Levels  
293 Kelvin Cross Sections

100-Fm-256



11

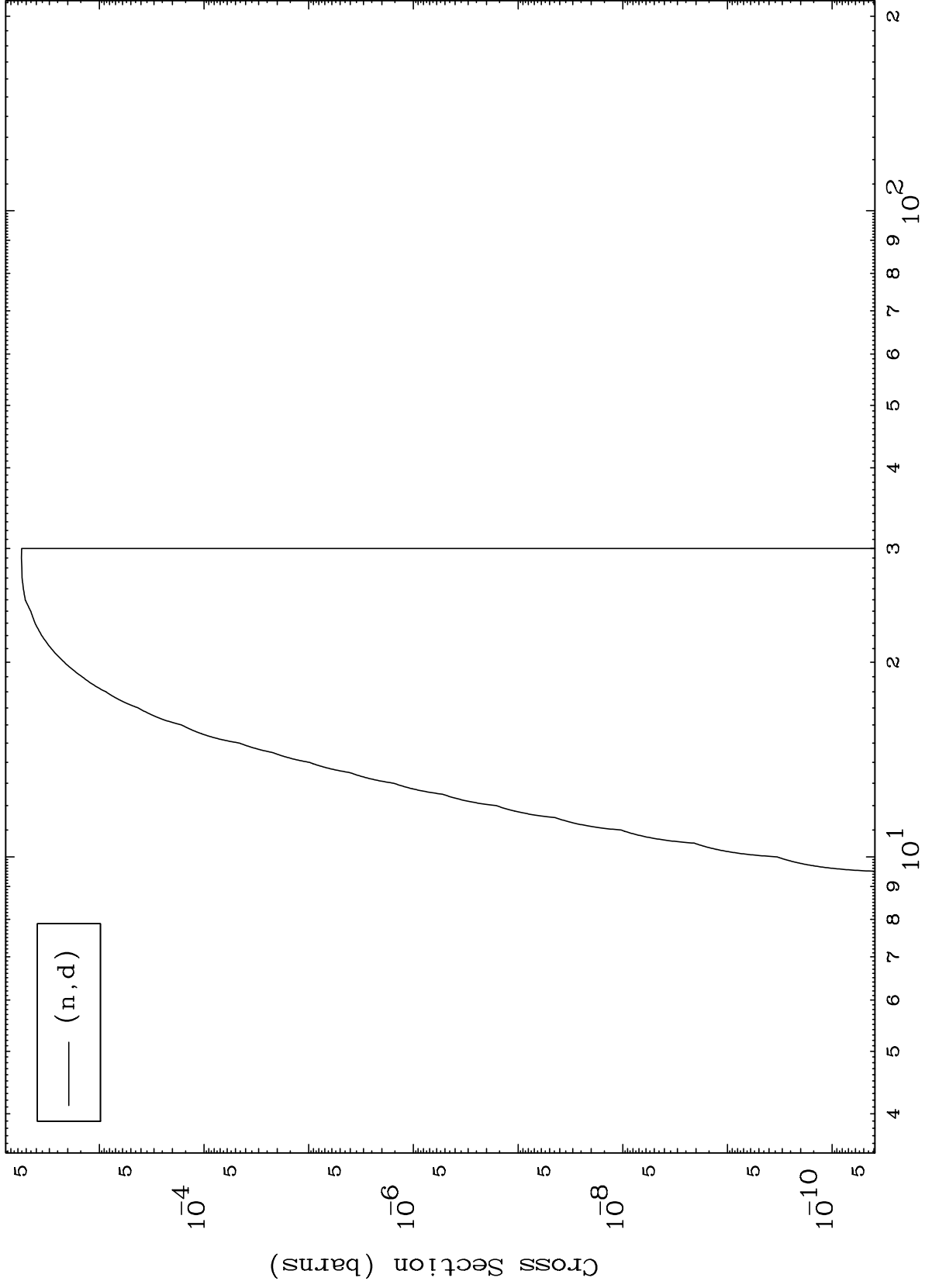
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-256



12

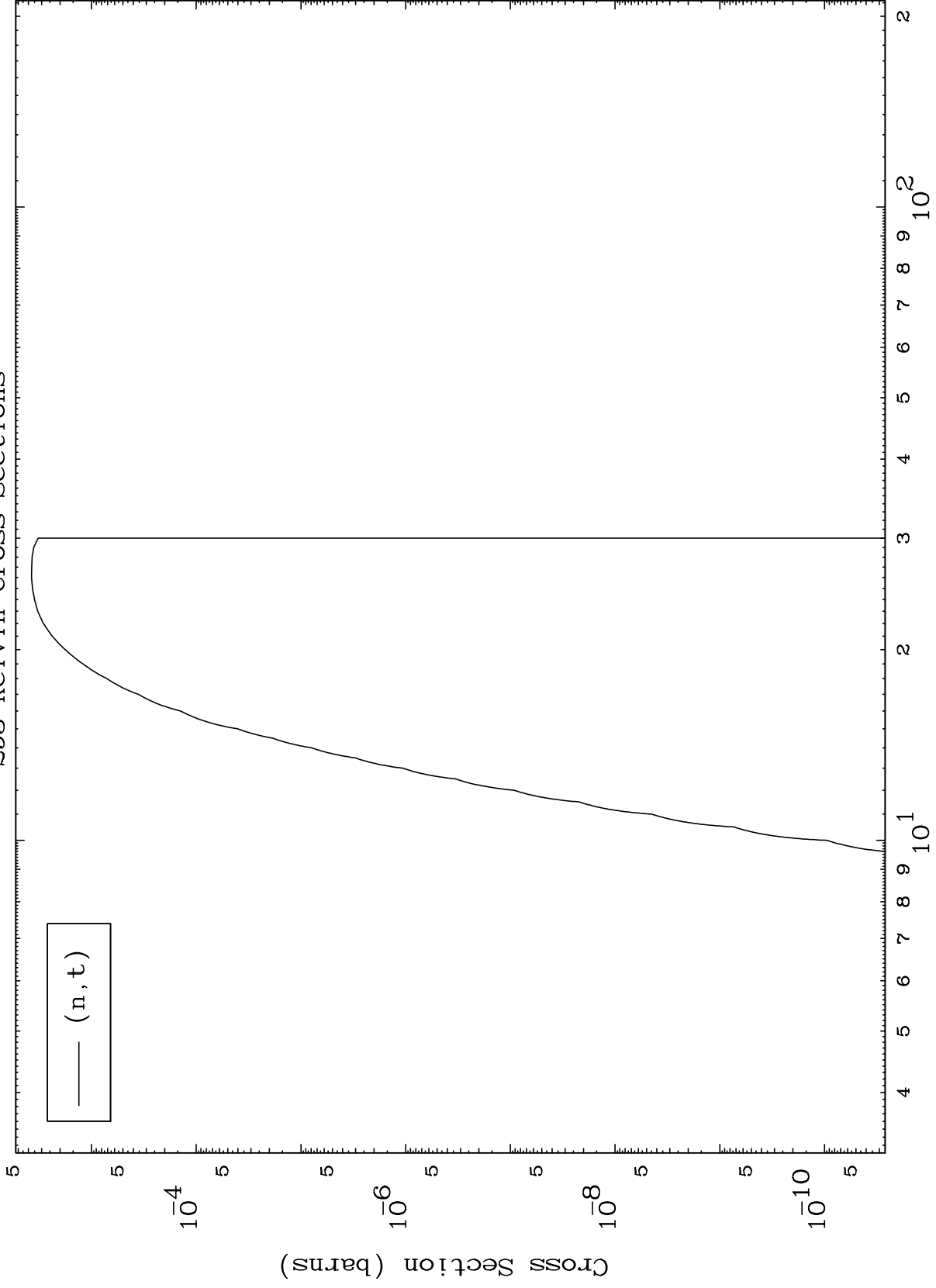
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,t) Levels  
293 Kelvin Cross Sections

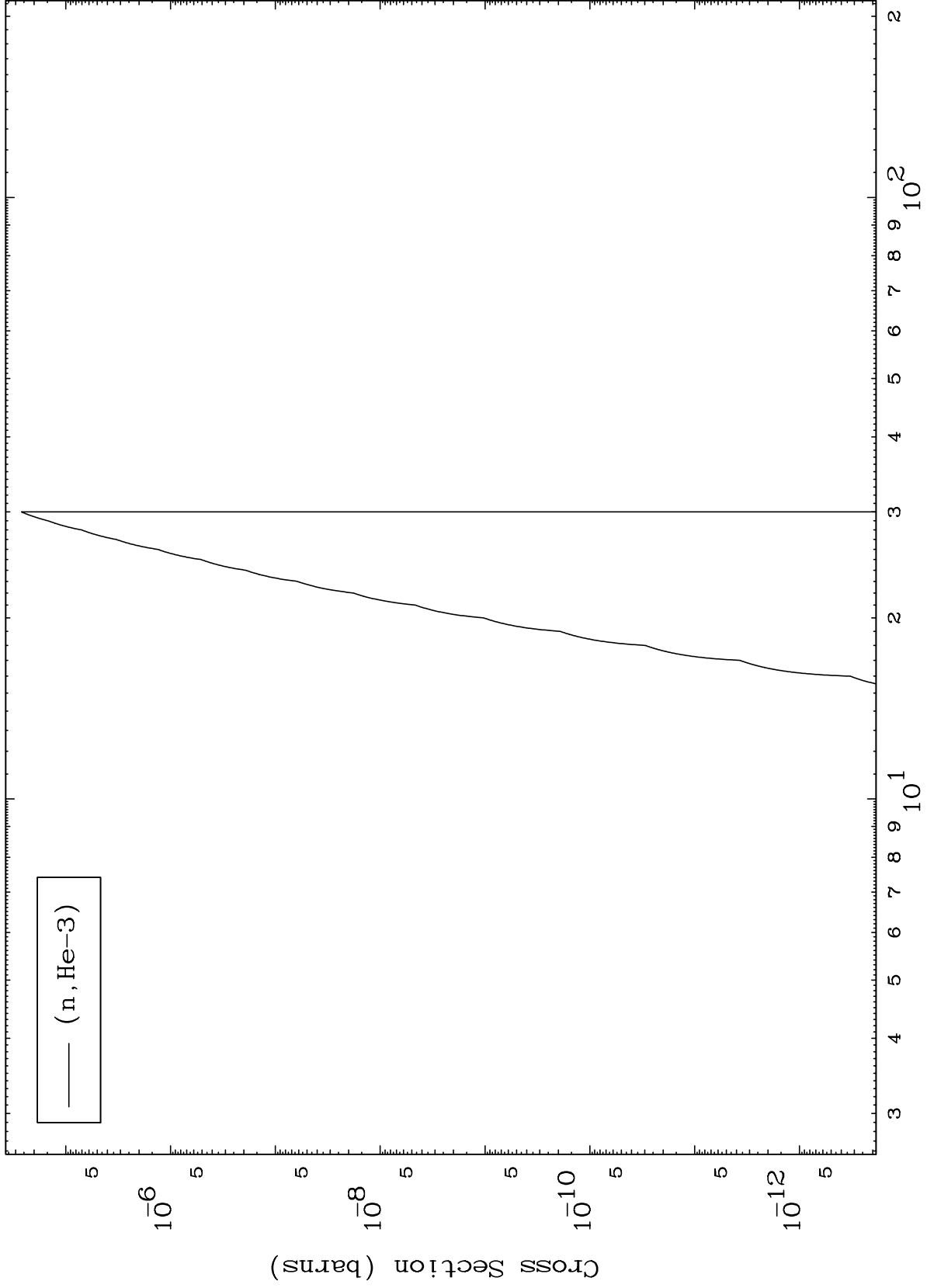
100-Fm-256



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(n,He3) Levels  
293 Kelvin Cross Sections

100-Fm-256



14

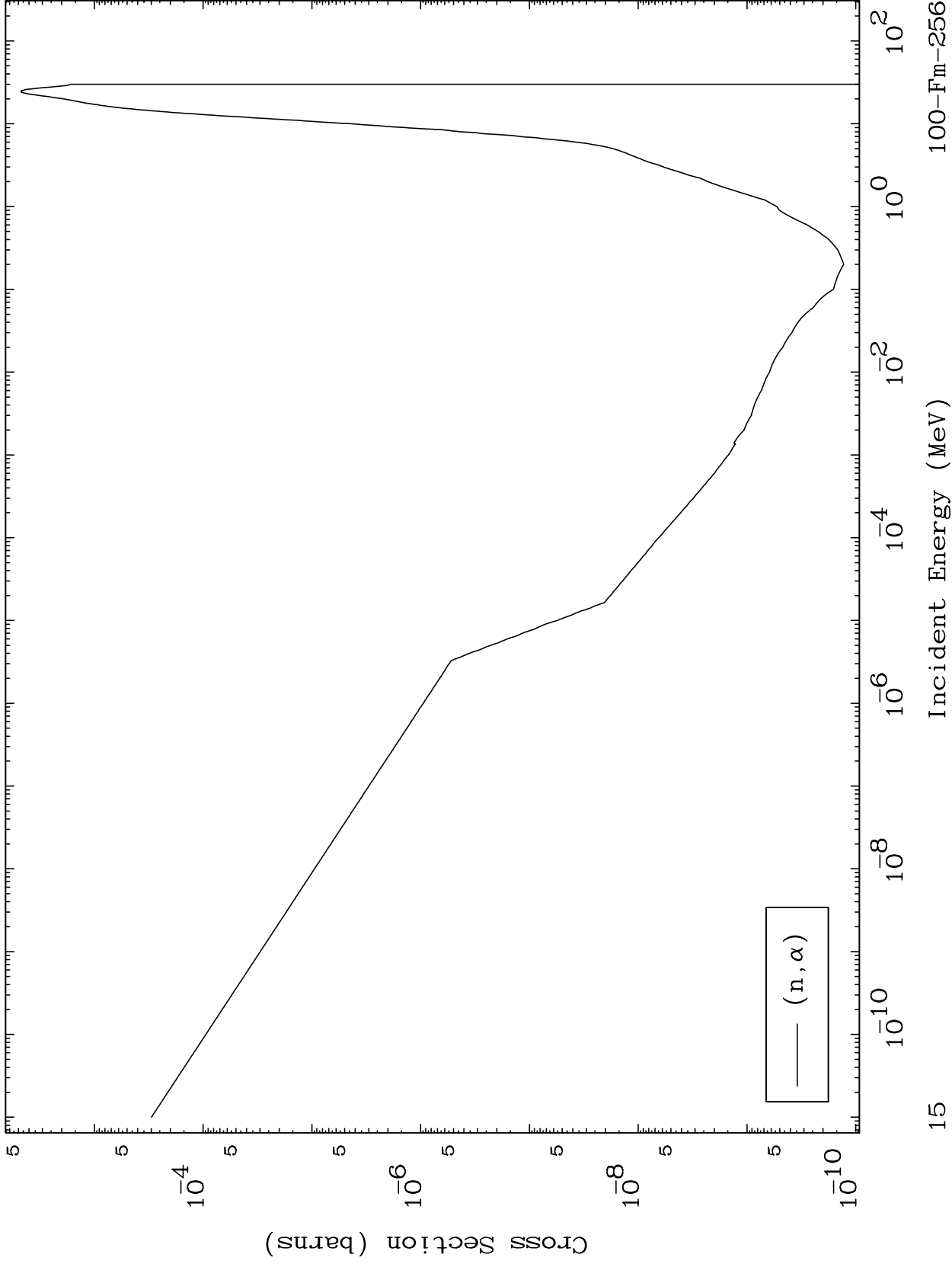
Incident Energy (MeV)

100-Fm-256

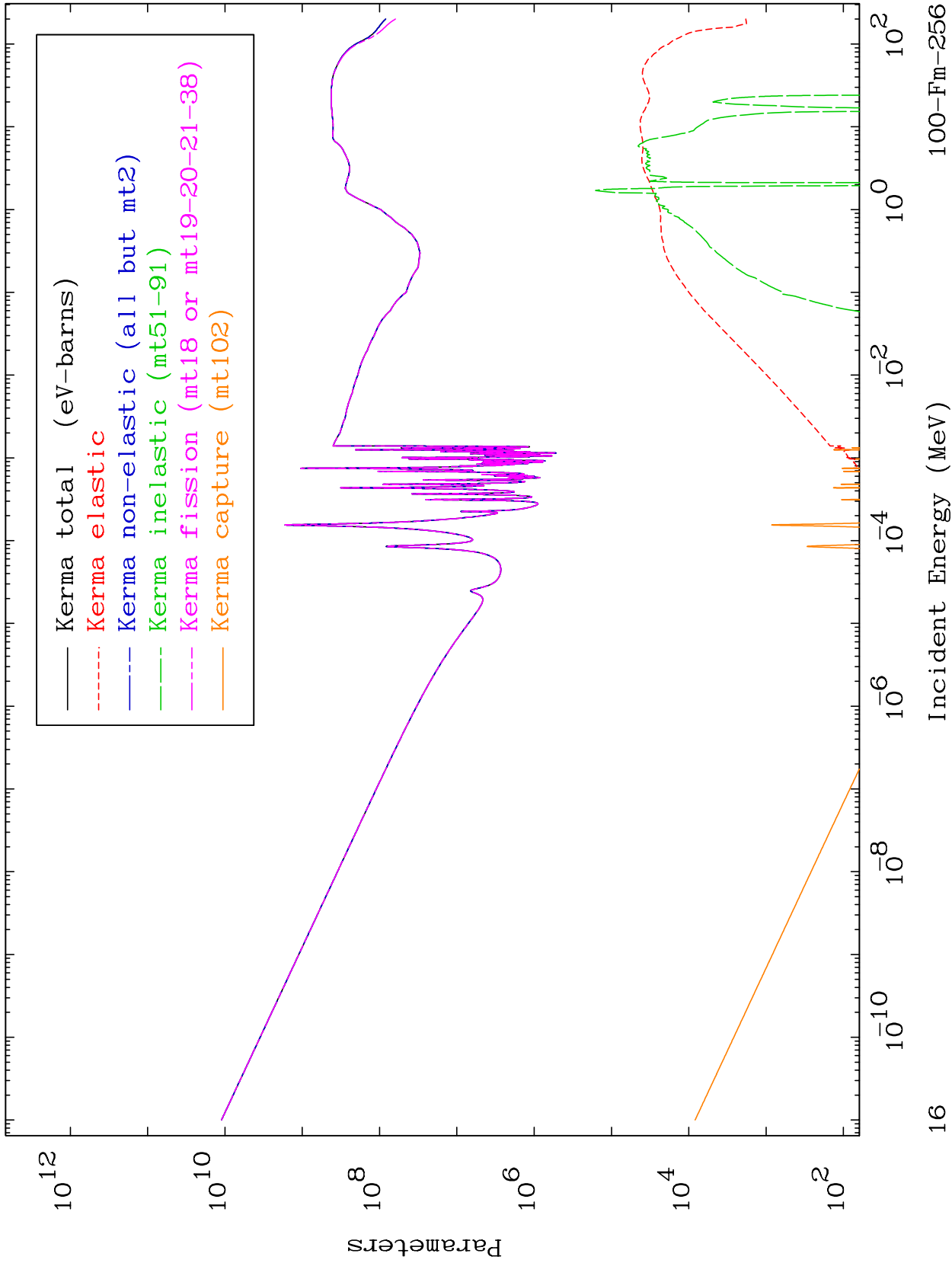
MAT 9937

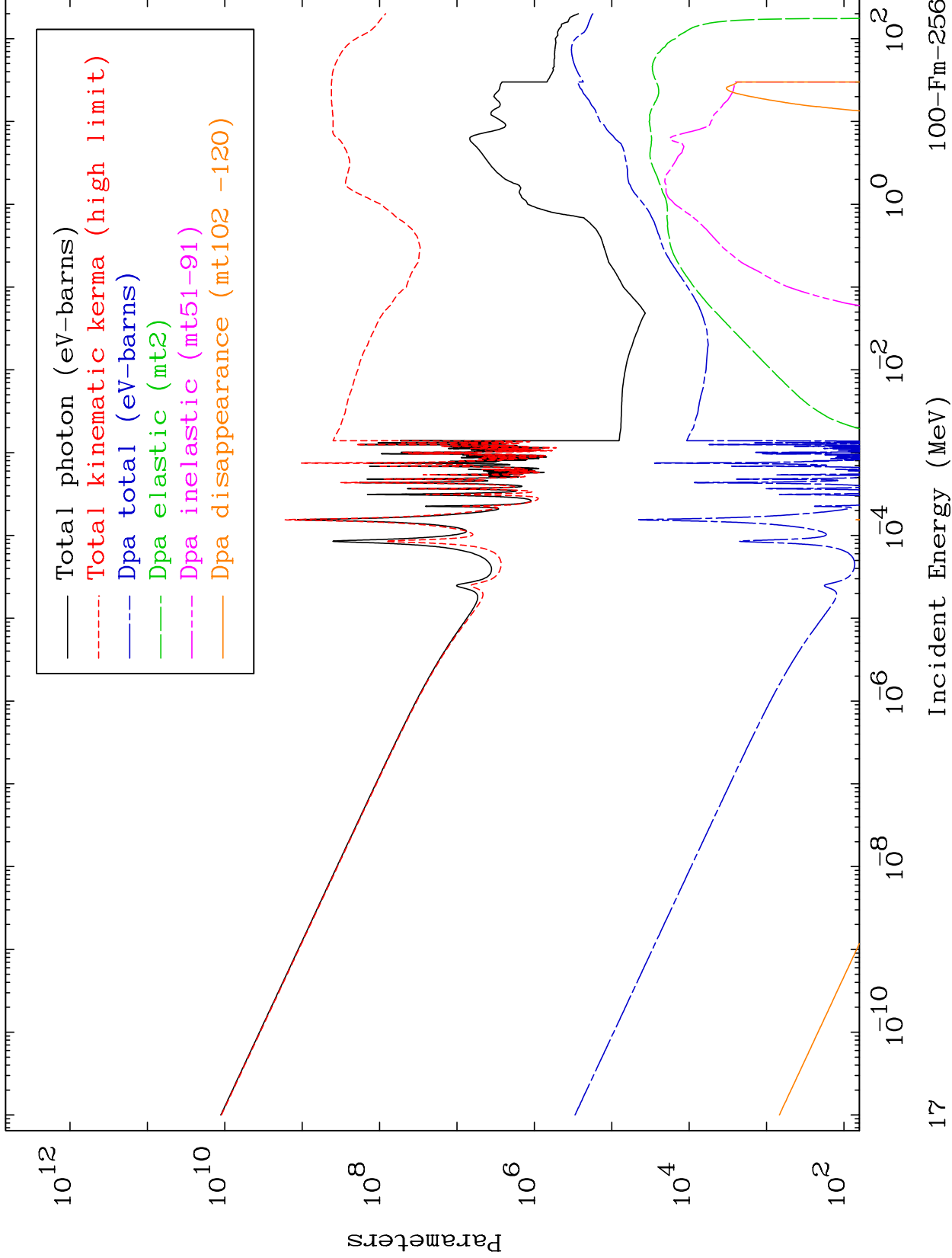
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

100-Fm-256





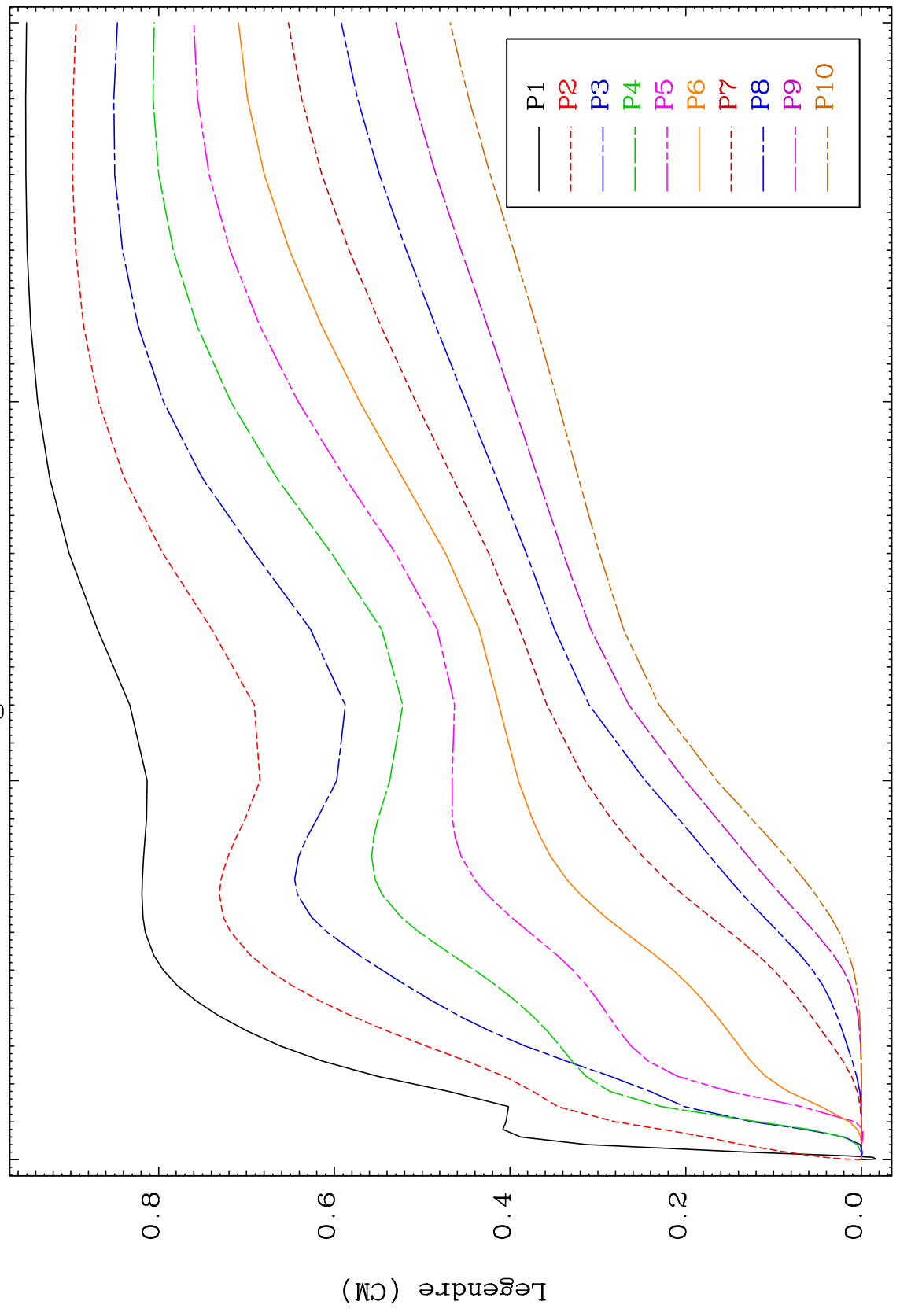




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Elastic Legendre Coefficients

100-Fm-256



18

Incident Energy (MeV)

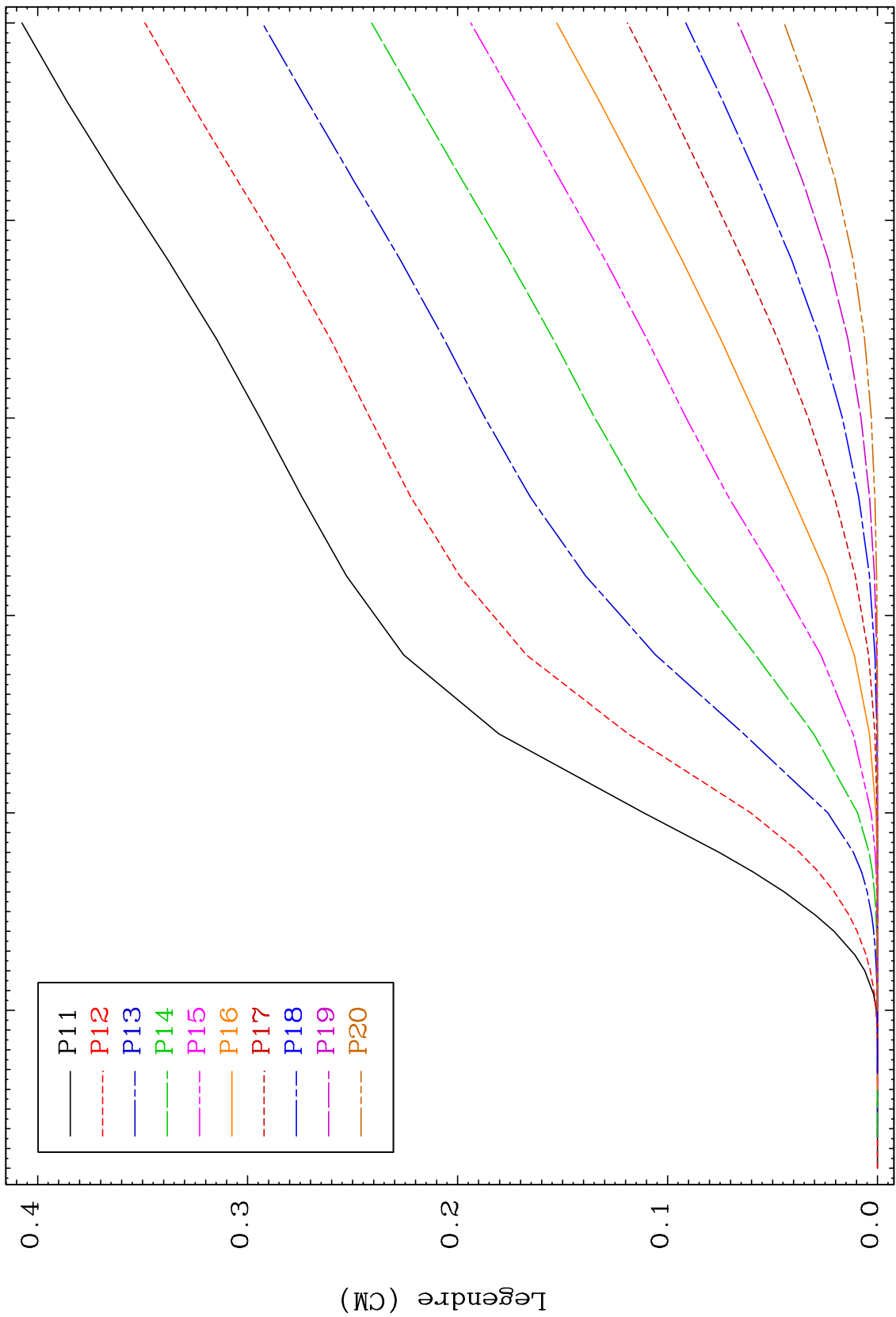
100-Fm-256

30

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Elastic Legendre Coefficients

100-Fm-256



19

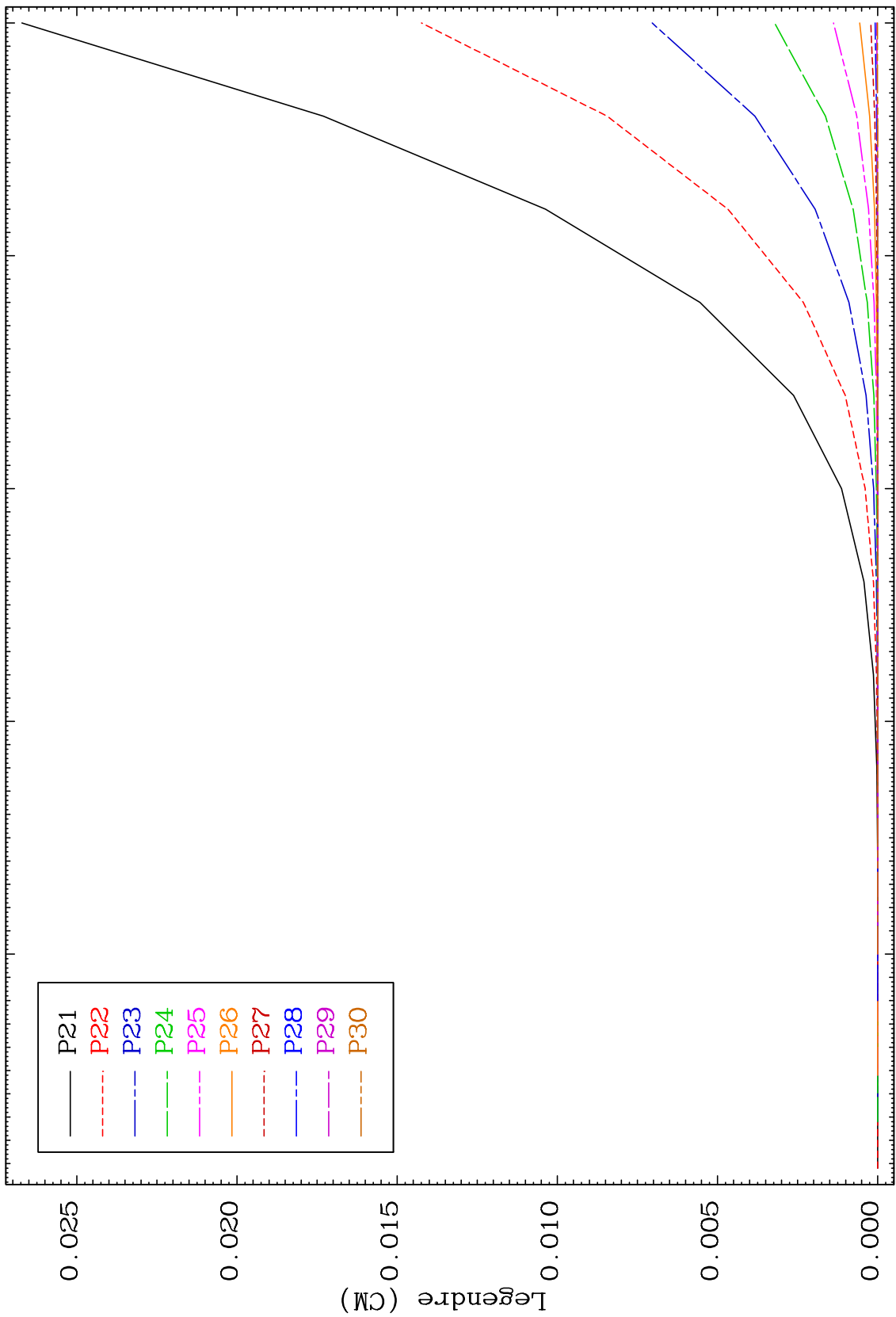
Incident Energy (MeV)

100-Fm-256

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Elastic Legendre Coefficients

100-Fm-256



20

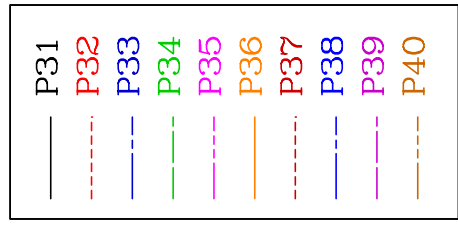
Incident Energy (MeV)

100-Fm-256

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Elastic Legendre Coefficients

100-Fm-256



$\times 10^{-6}$

3

Legendre (CM)

2

1

0

15

20

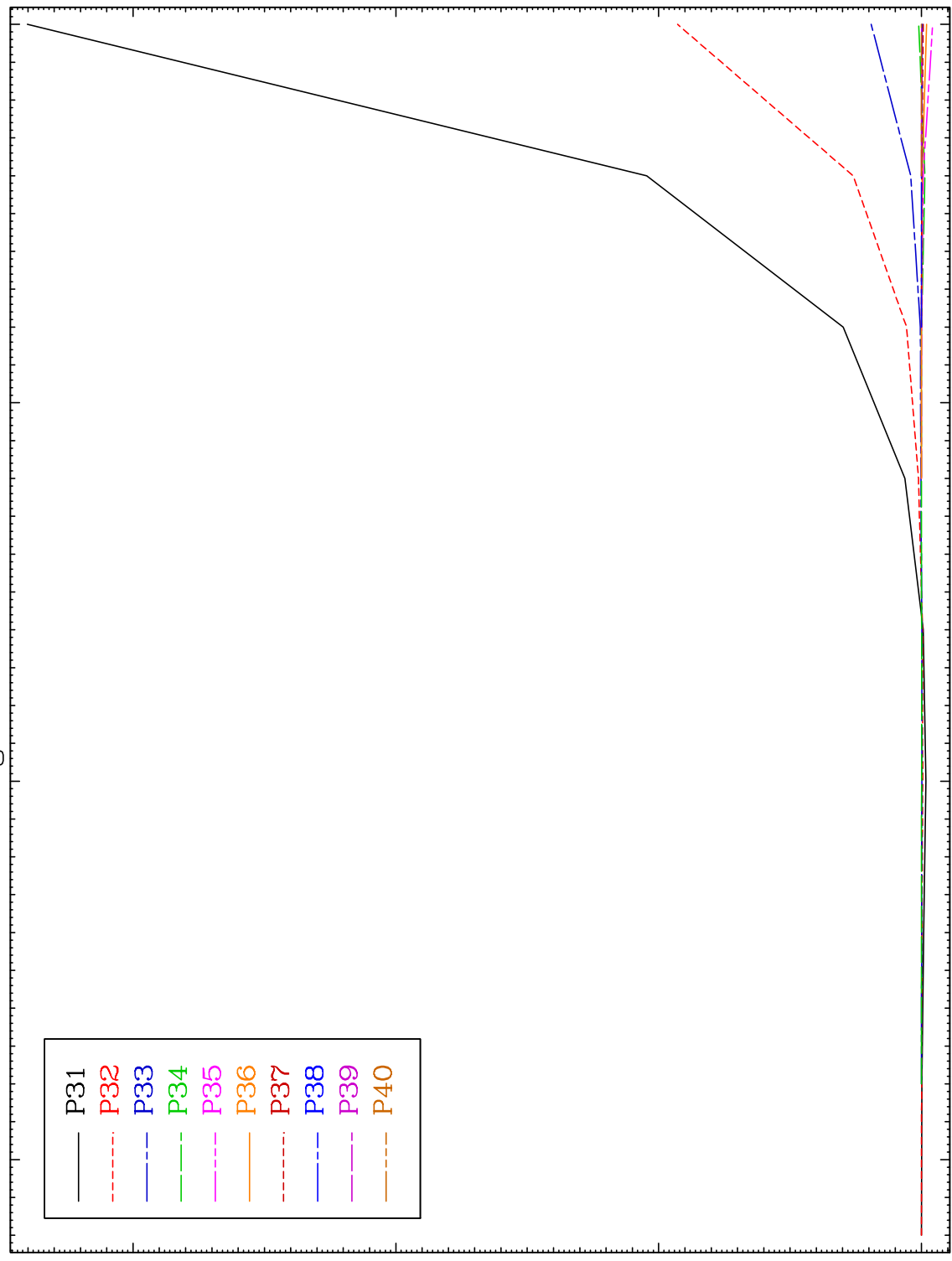
25

30

21

Incident Energy (MeV)

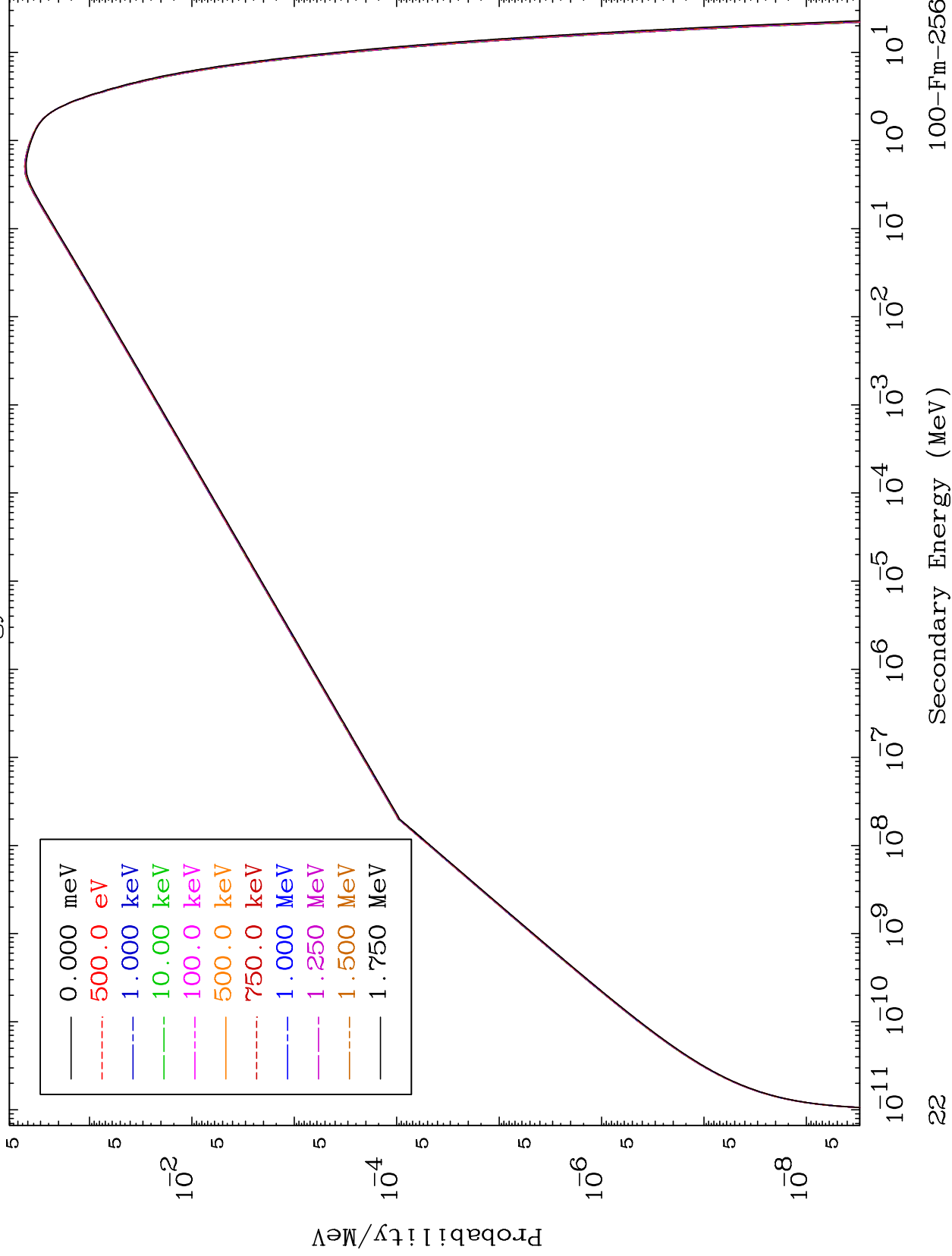
100-Fm-256



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### Fission Energy Distribution

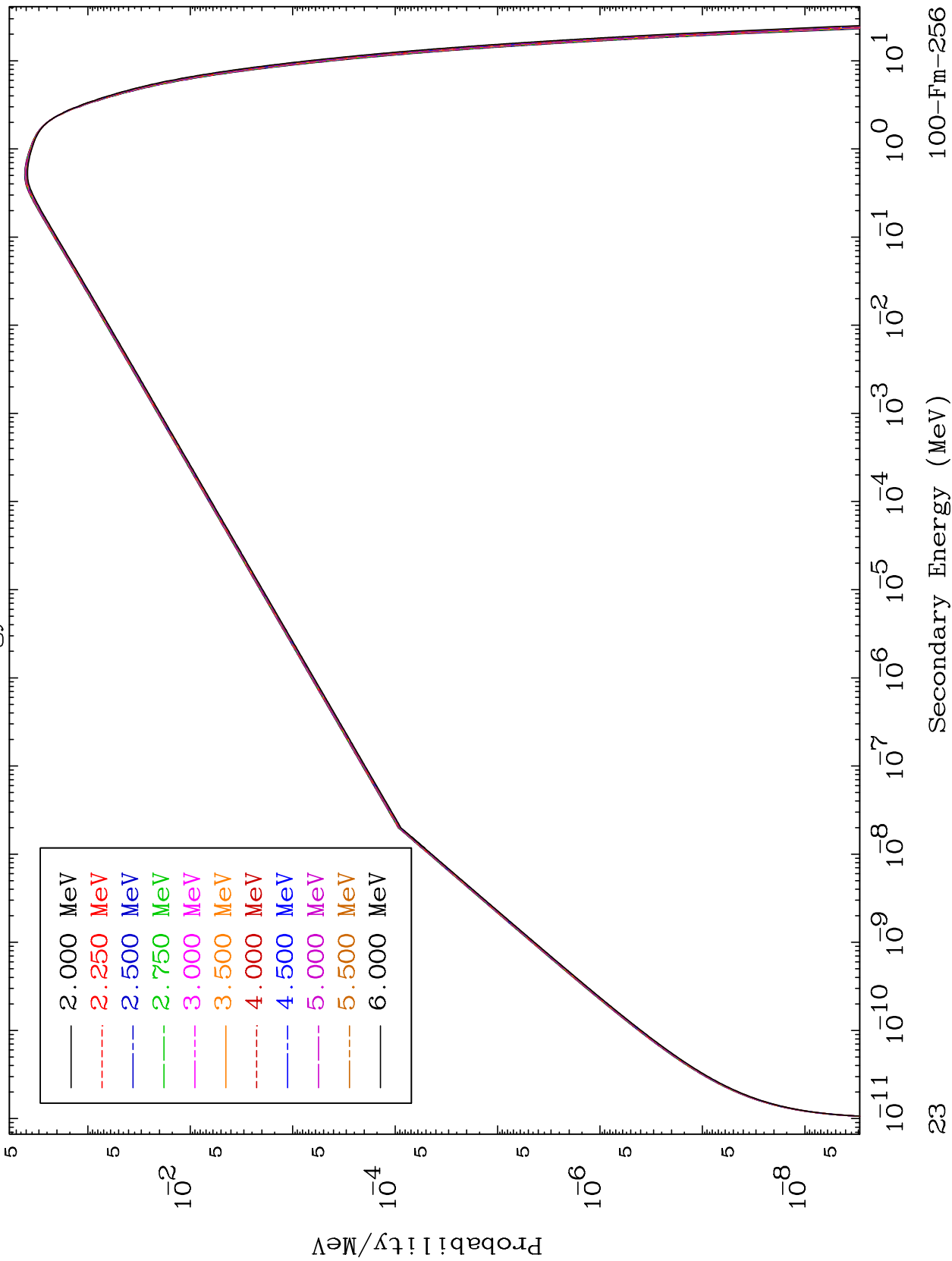
100-Fm-256



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### Fission Energy Distribution

100-Fm-256

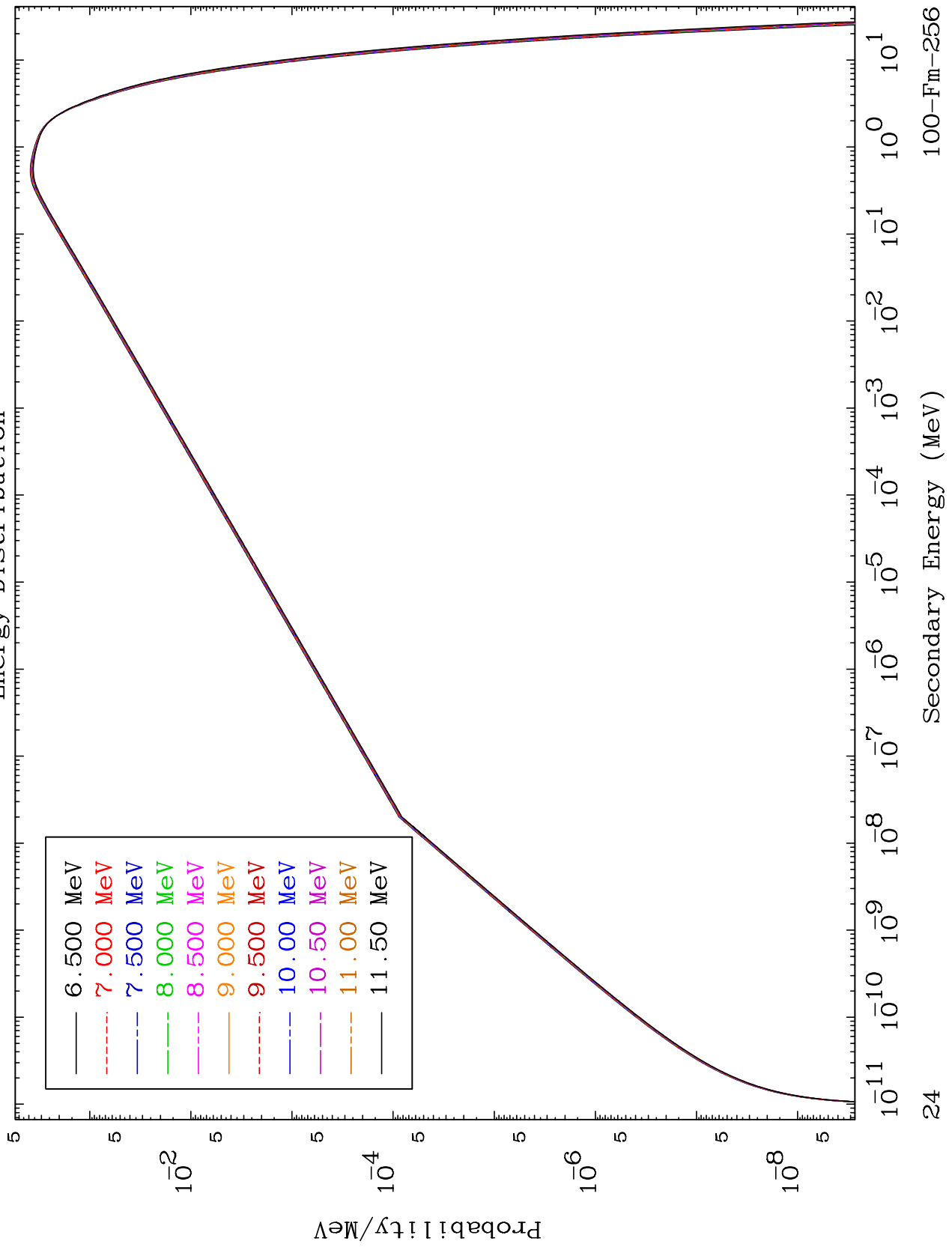




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### Fission Energy Distribution

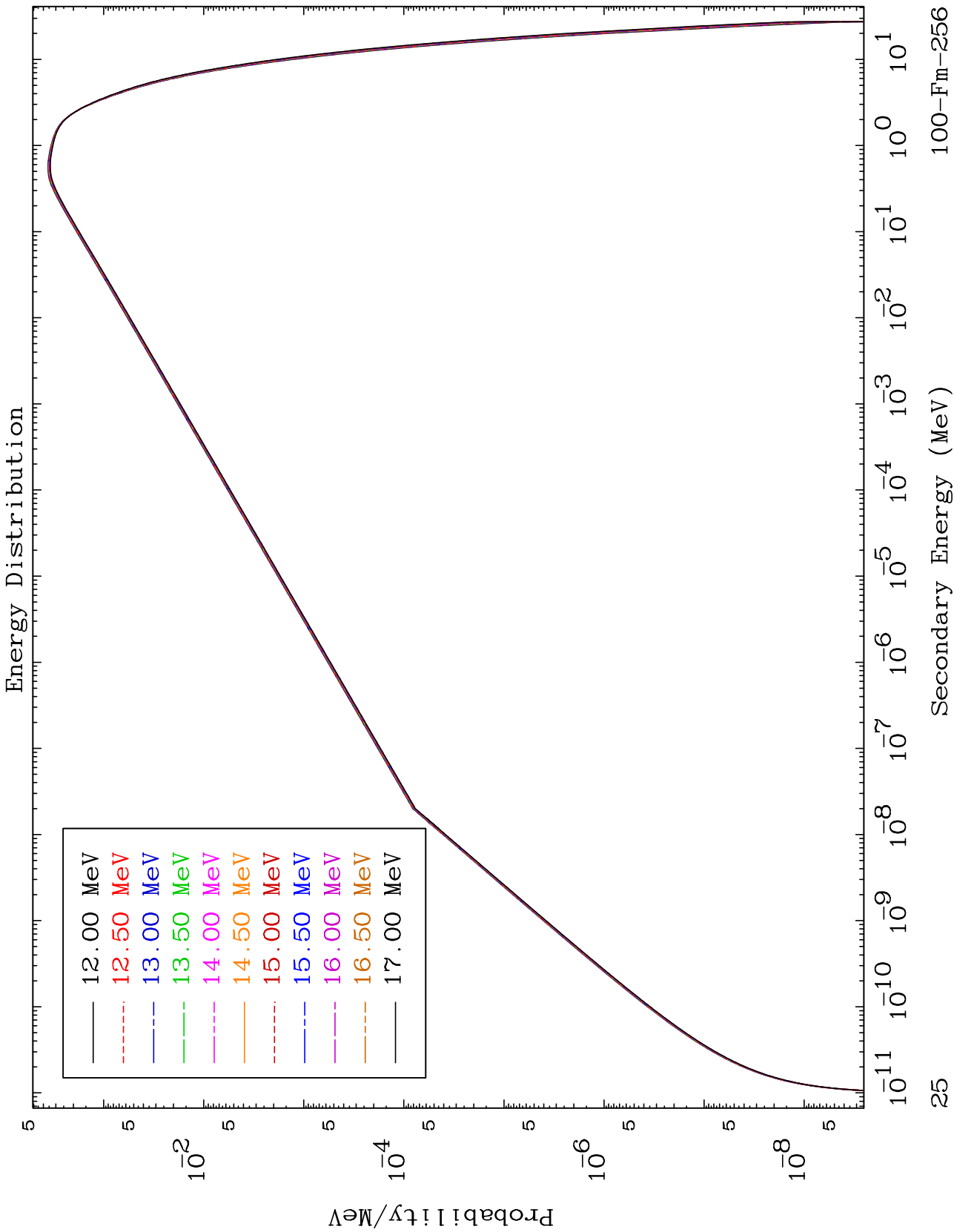
100-Fm-256



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Fission

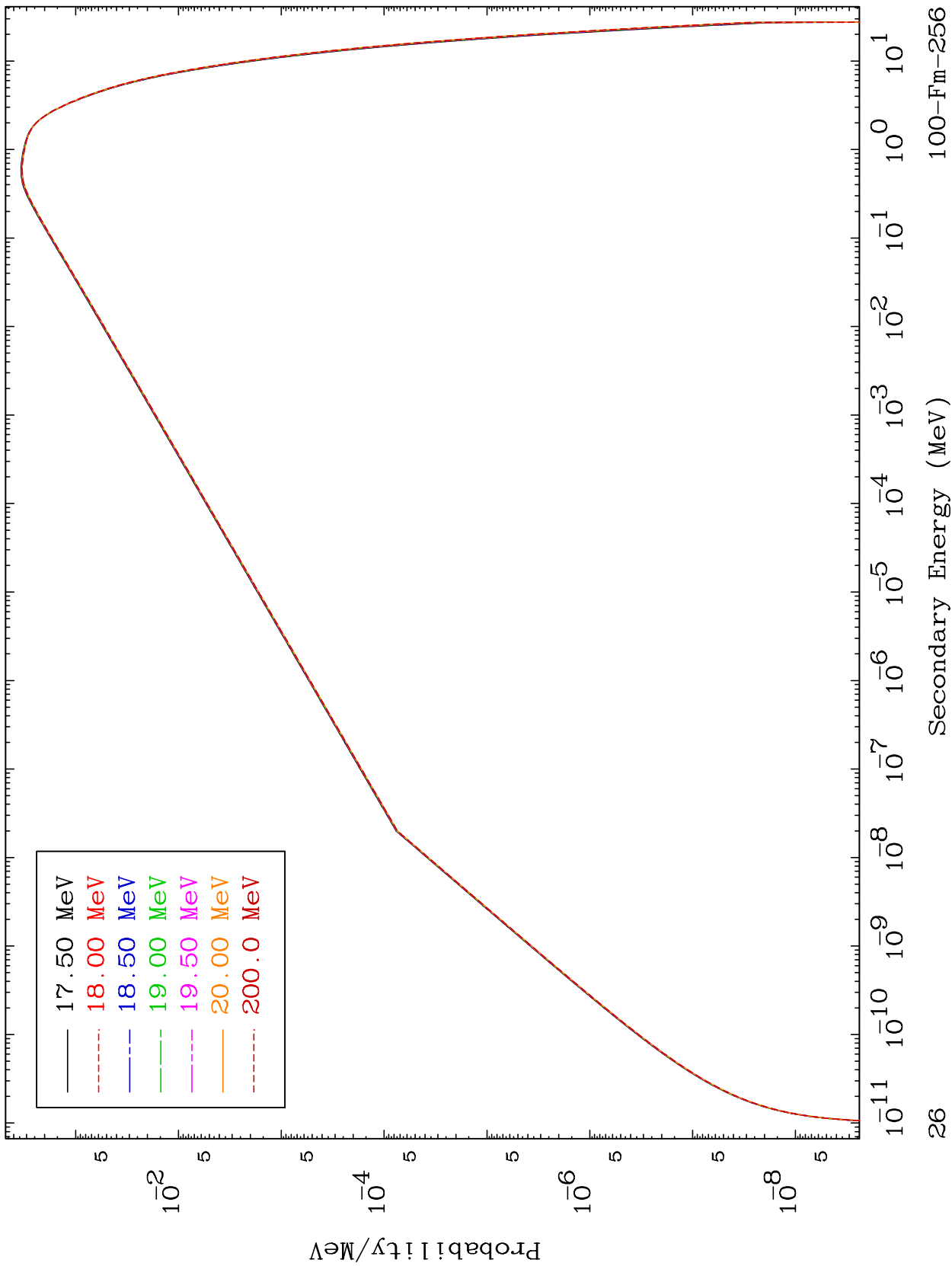
100-Fm-256



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Fission Energy Distribution

100-Fm-256

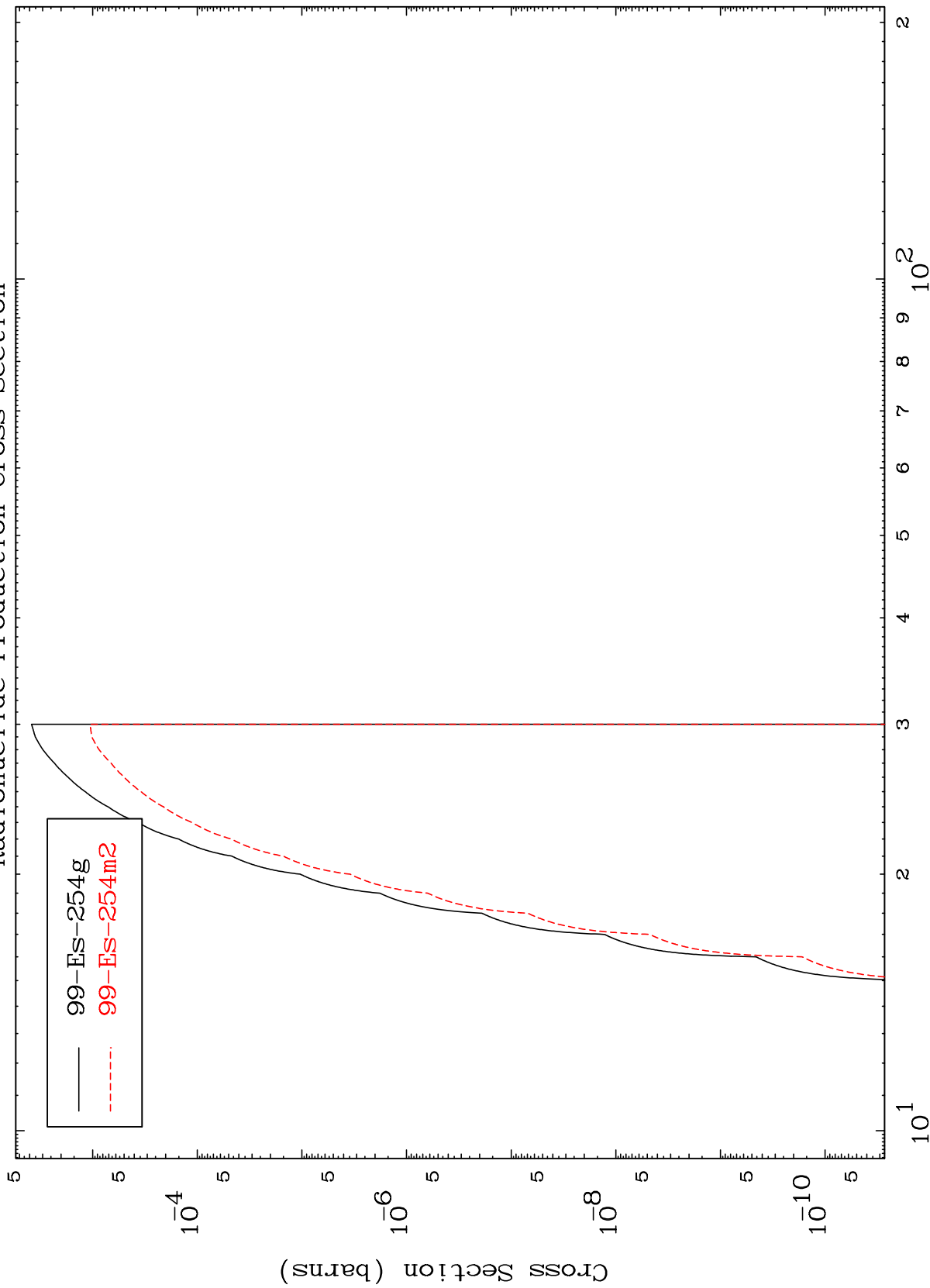


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(n,n') d

100-Fm-256

Radionuclide Production Cross Section



27

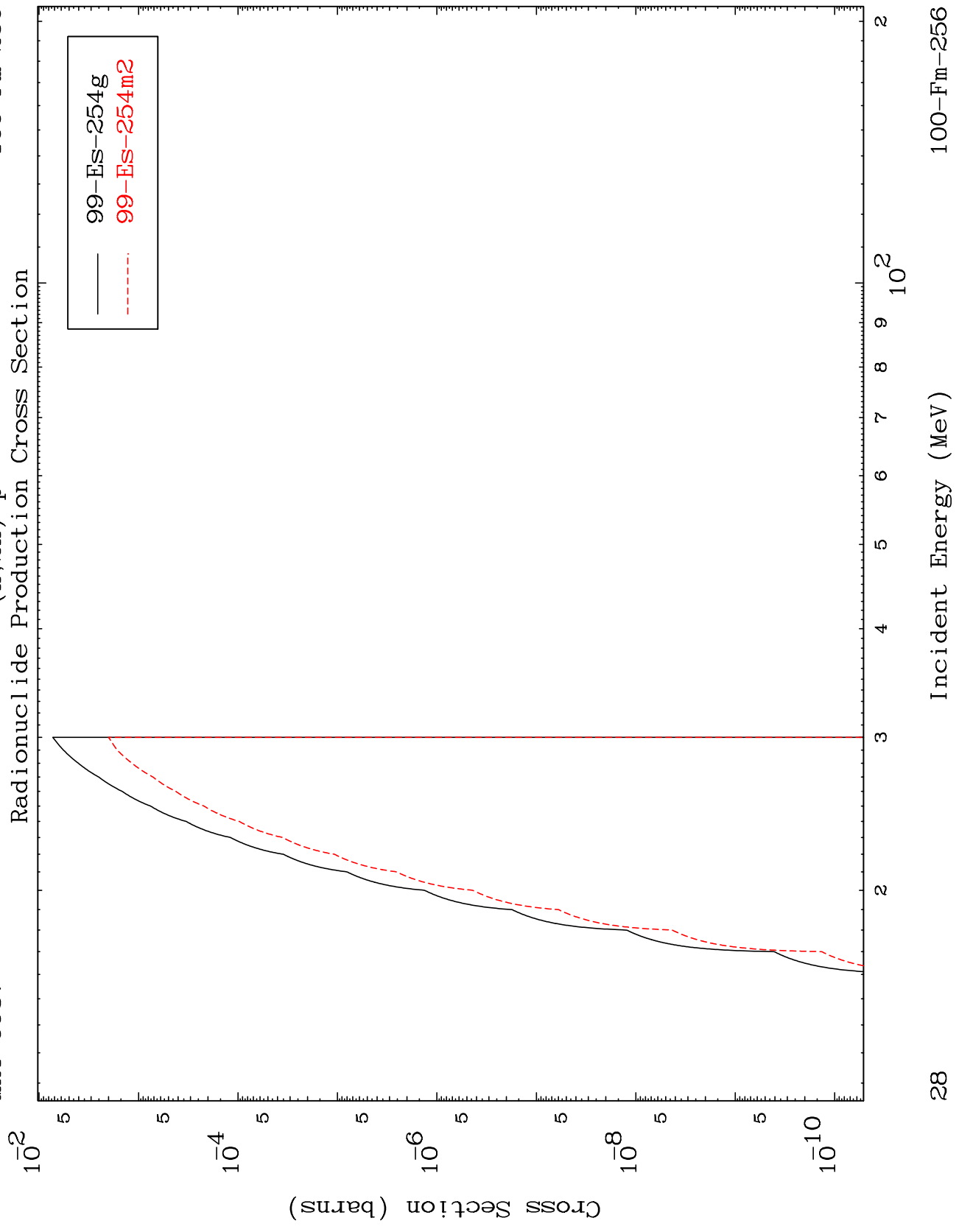
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,2n) p

100-Fm-256



28

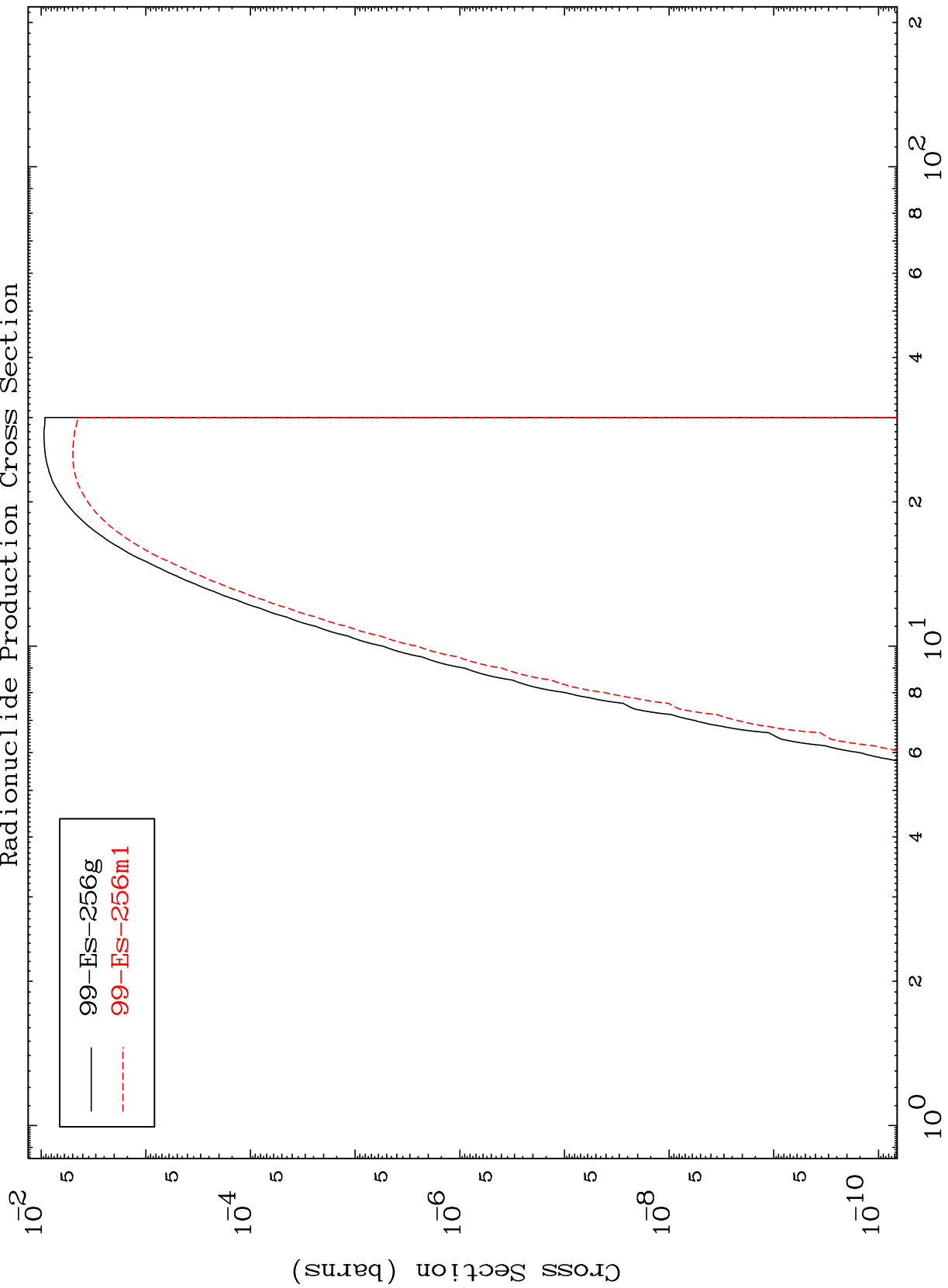
Incident Energy (MeV)

100-Fm-256

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100-Fm-256

Radionuclide Production Cross Section (n,p)



99-Es-256g  
99-Es-256m1

Incident Energy (MeV)

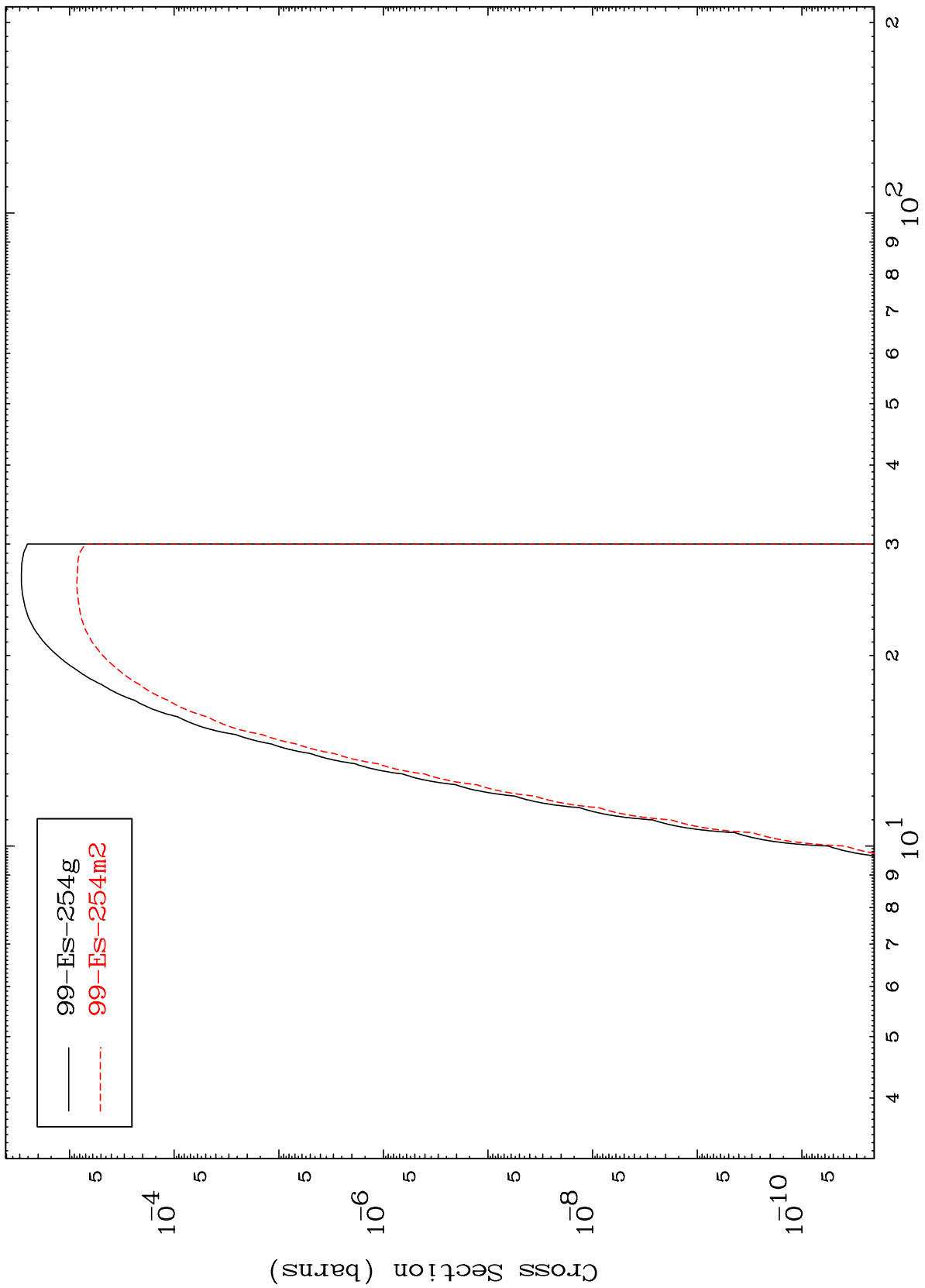
100-Fm-256

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100-Fm-256

(n, t)  
Radionuclide Production Cross Section



30

Incident Energy (MeV)

100-Fm-256