

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

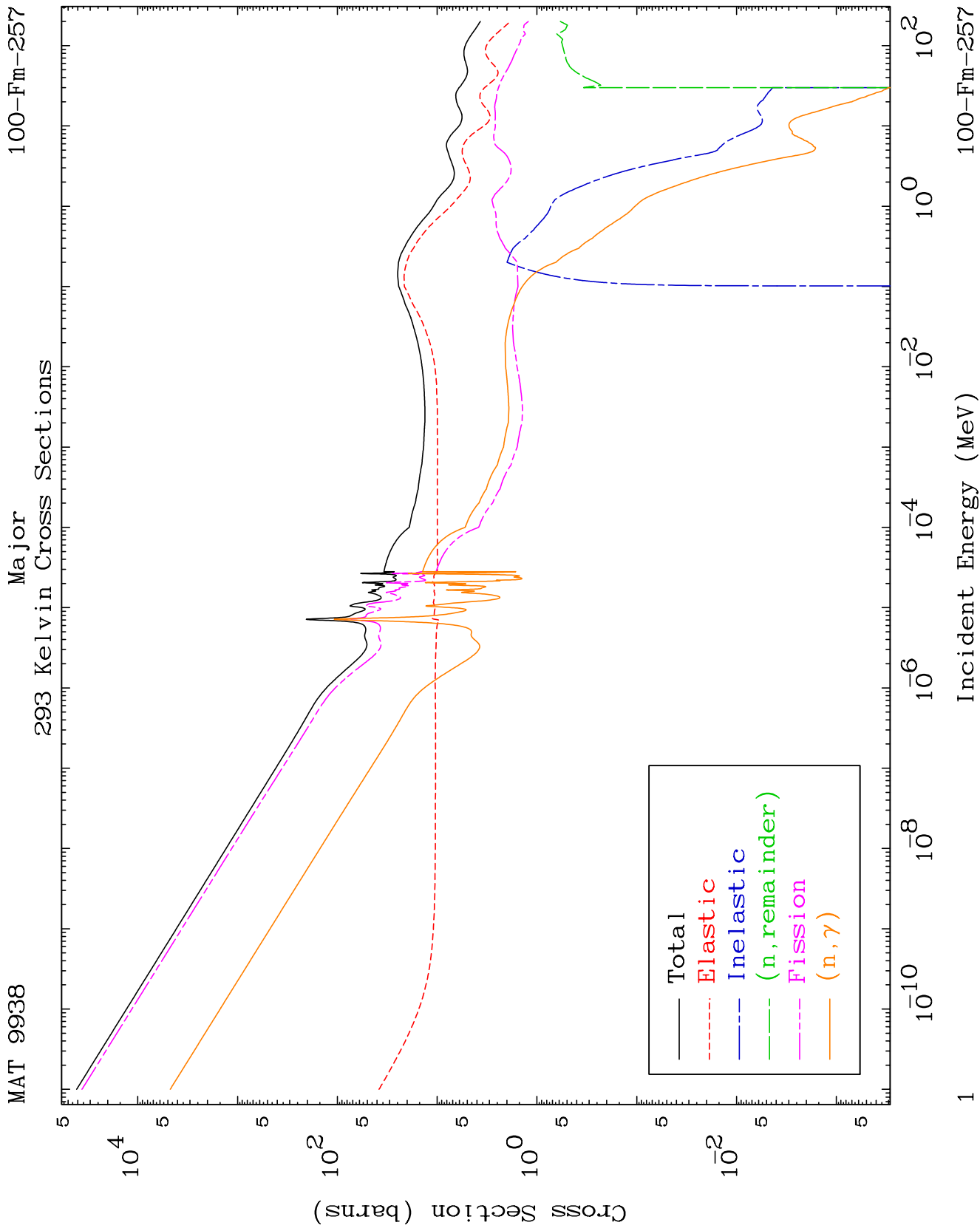
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Web:redcullen1.net/HOMEPAGE.NEW

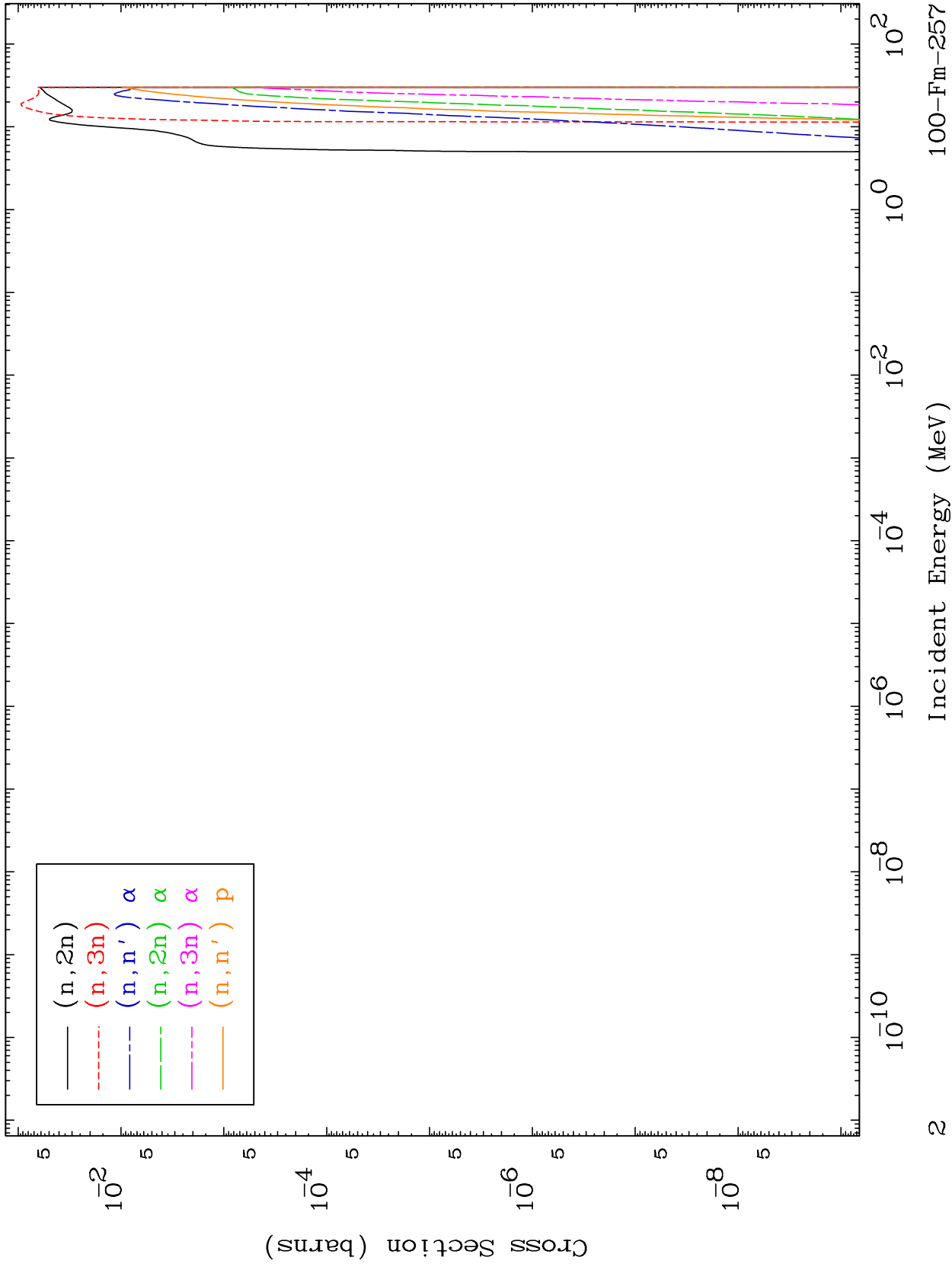
Press Mouse Button to Start



MAT 9938

Neutron Production
293 Kelvin Cross Sections

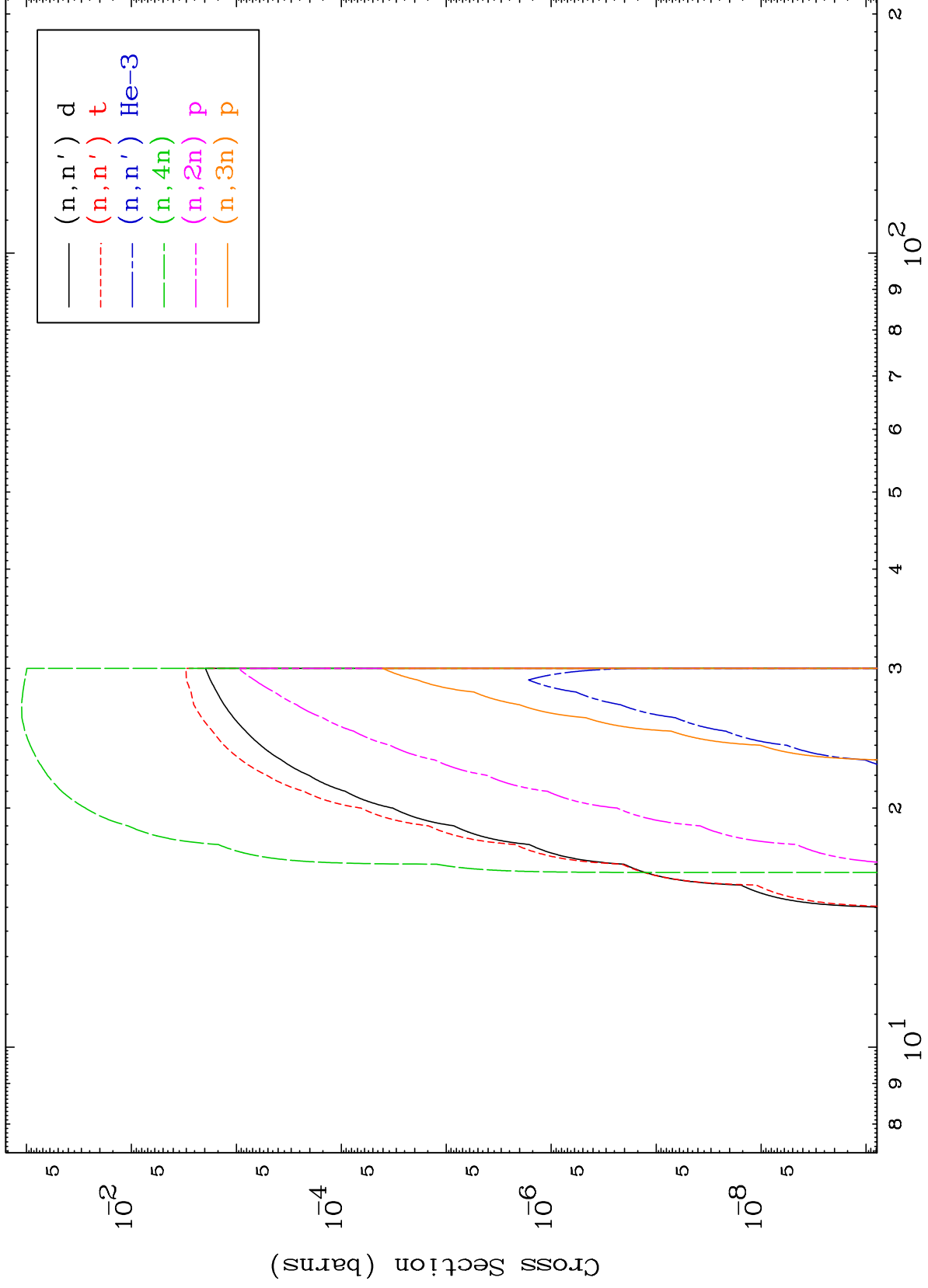
100-Fm-257



MAT 9938

Neutron Production
293 Kelvin Cross Sections

100-Fm-257



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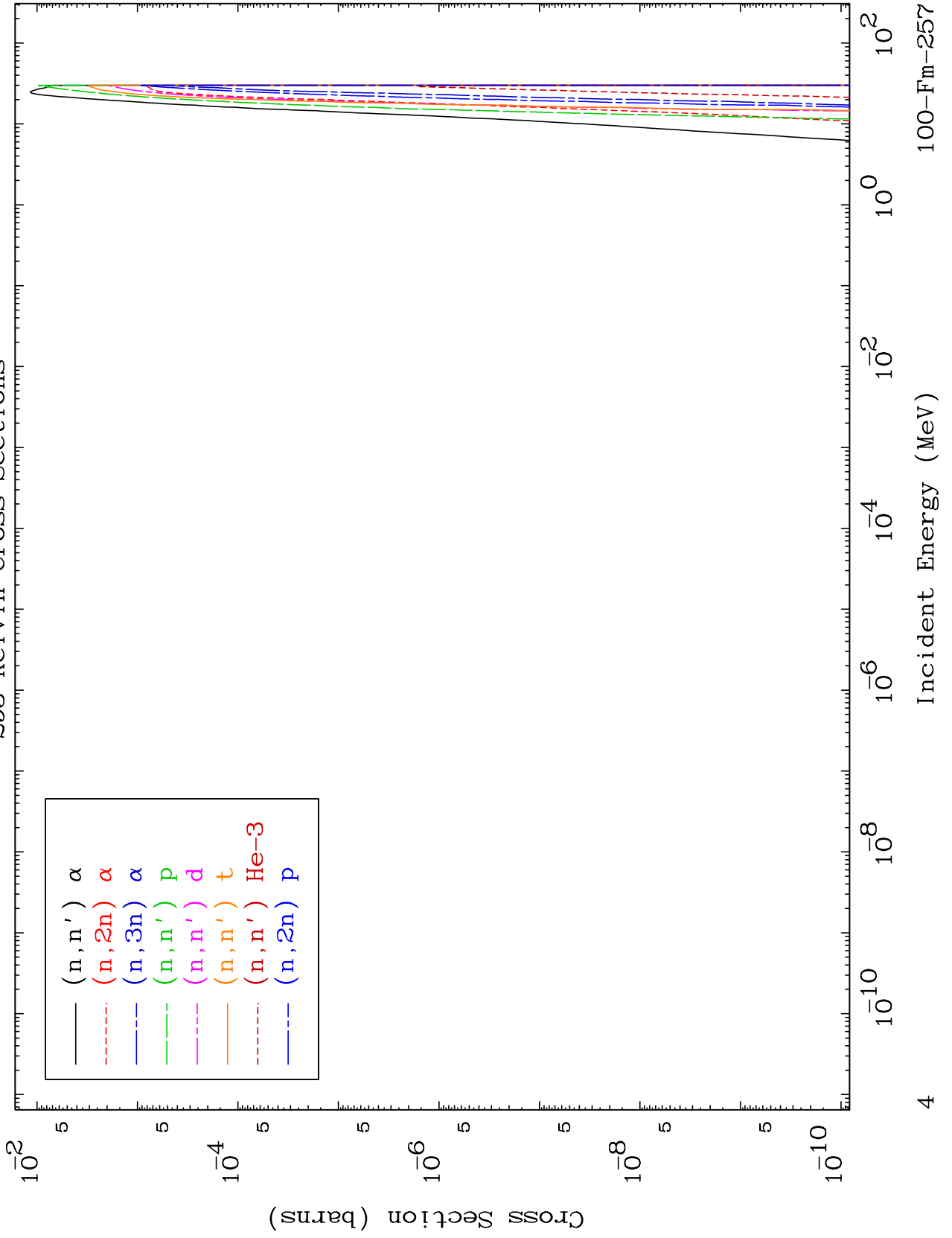
Incident Energy (MeV)

100-Fm-257

MAT 9938

Charged Particle
293 Kelvin Cross Sections

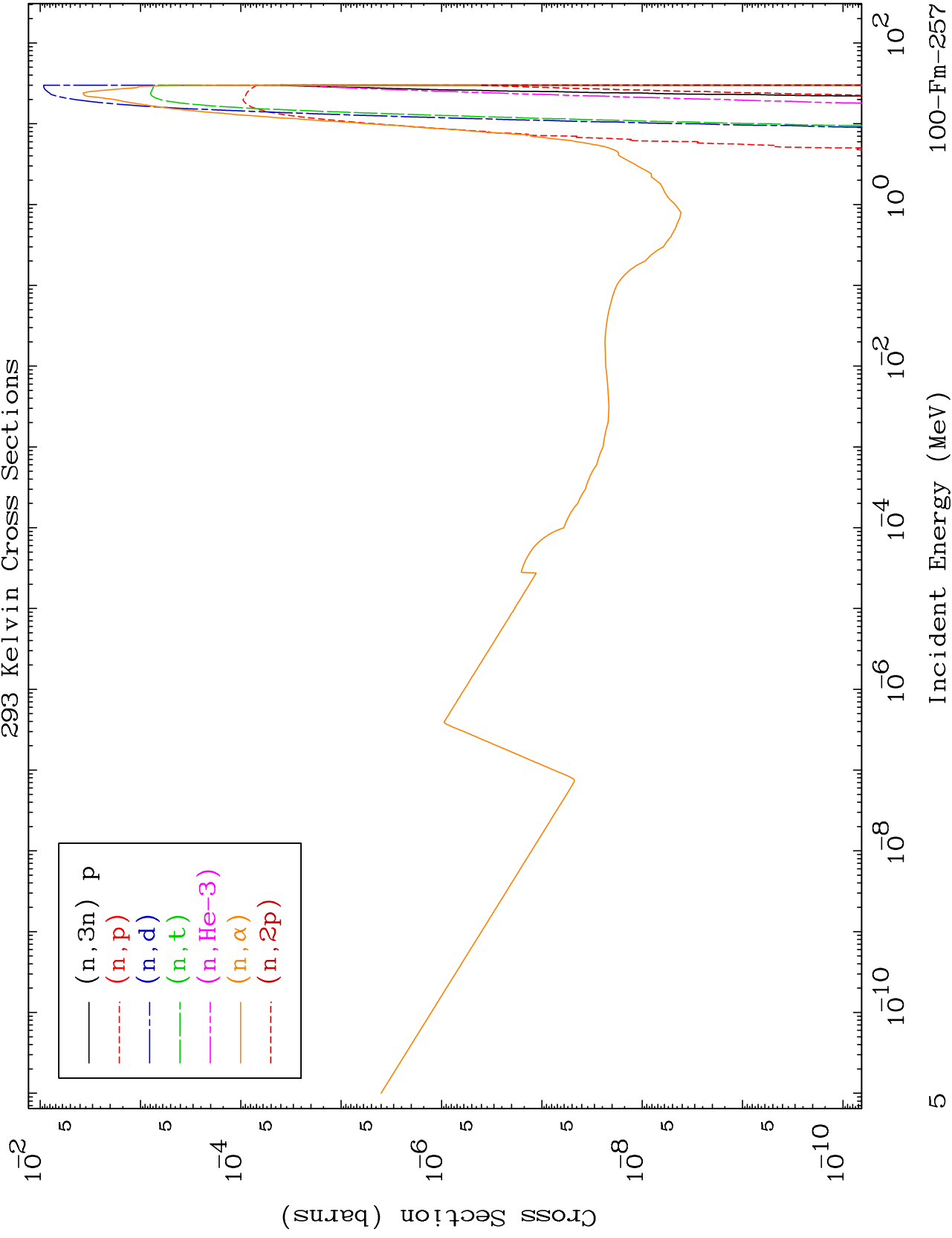
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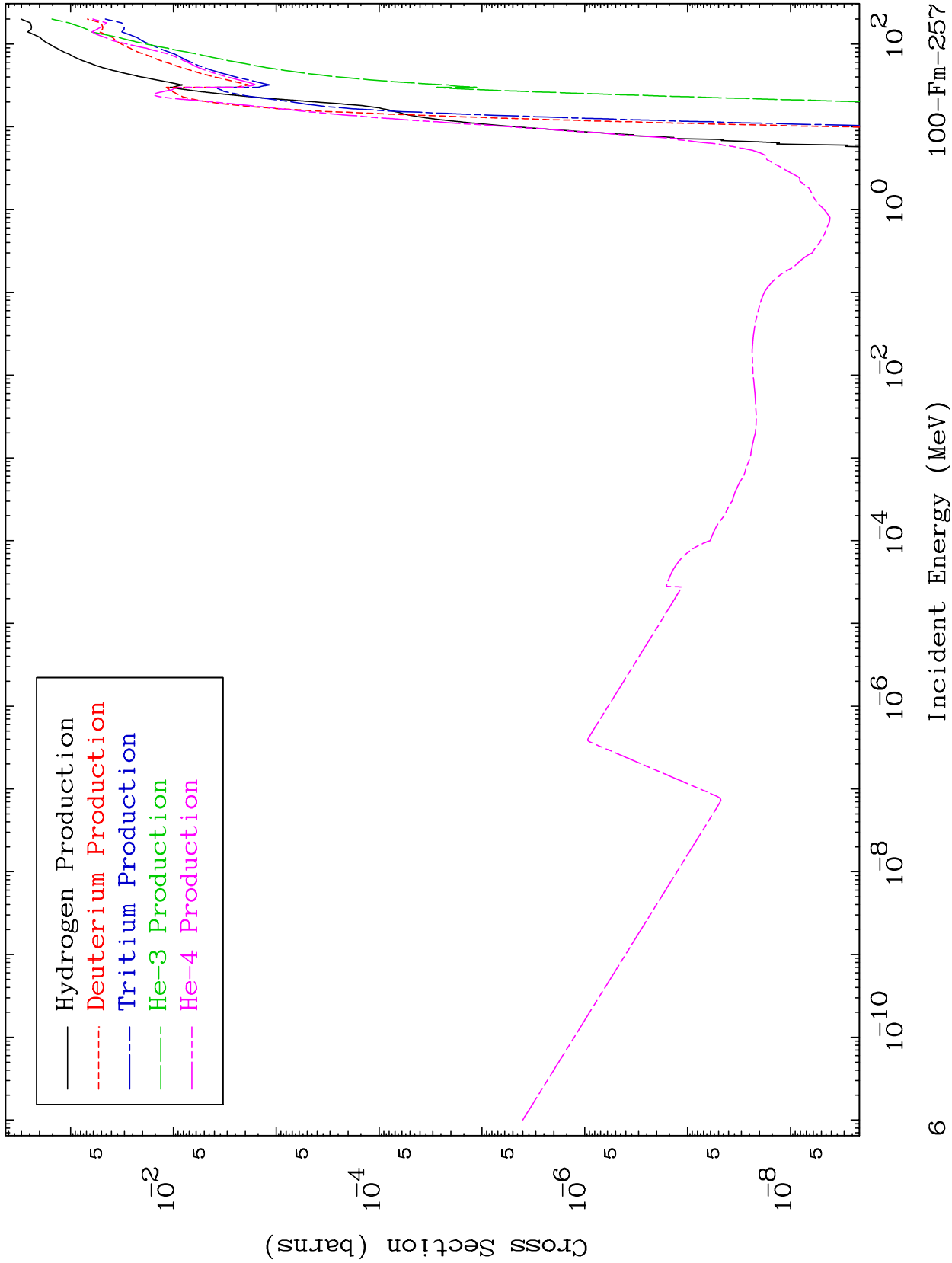
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Charged Particle
293 Kelvin Cross Sections

100-Fm-257



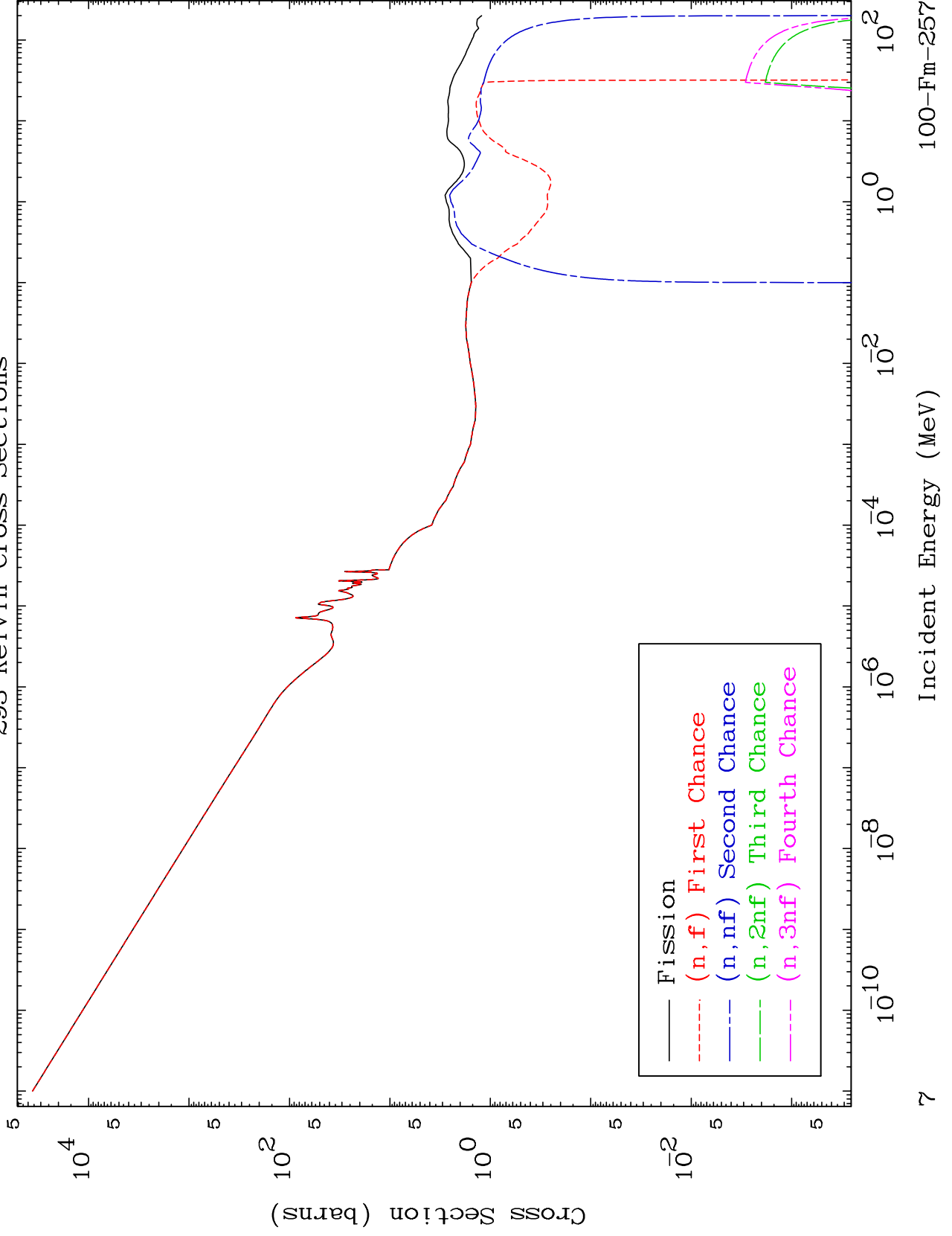
5



MAT 9938

Fission
293 Kelvin Cross Sections

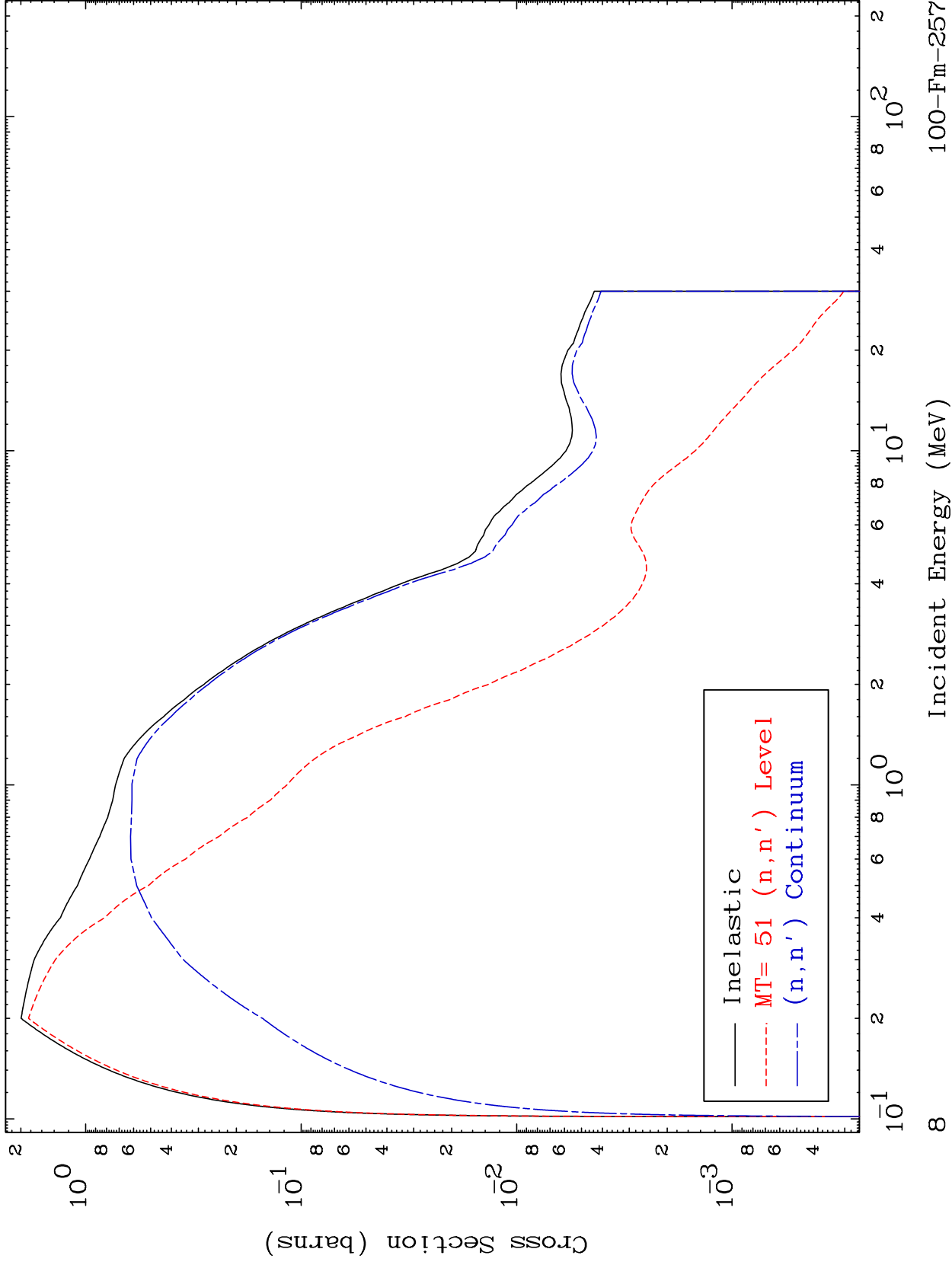
100-Fm-257



MAT 9938

(n,n') Level
293 Kelvin Cross Sections

100-Fm-257



100-Fm-257

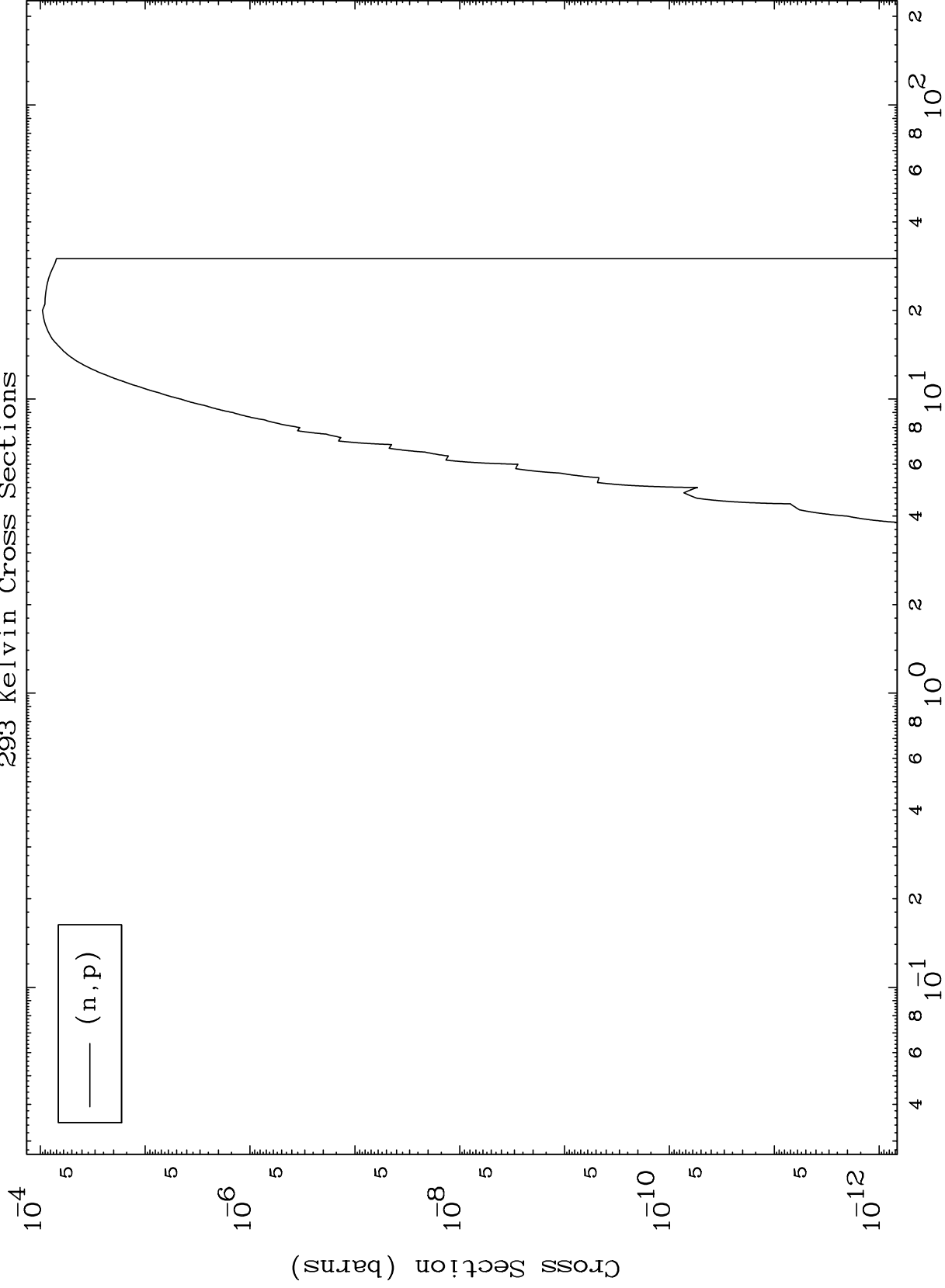
Incident Energy (MeV)

8

MAT 9938

(n,p) Levels
293 Kelvin Cross Sections

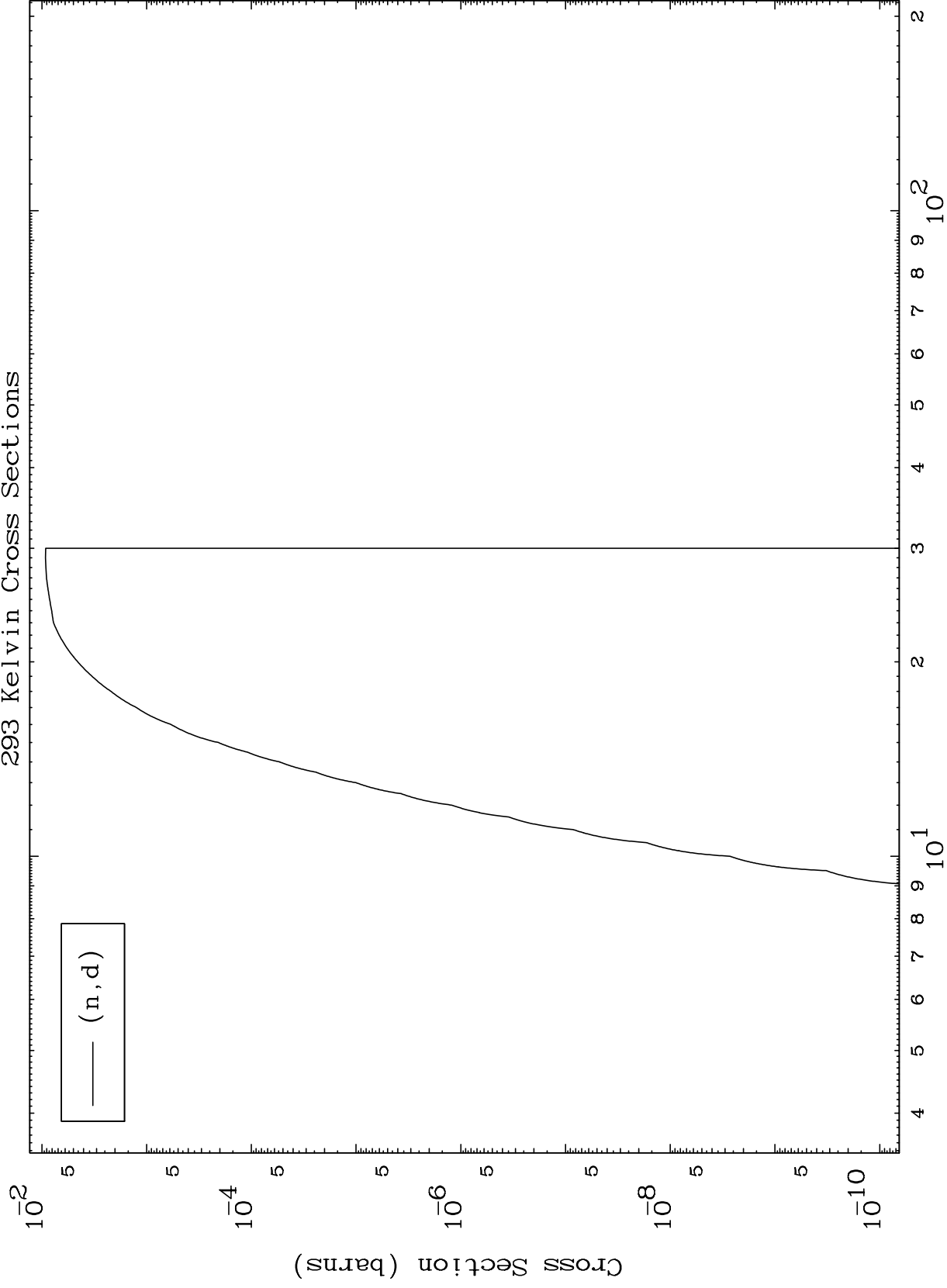
100-Fm-257



MAT 9938

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-257



100-Fm-257

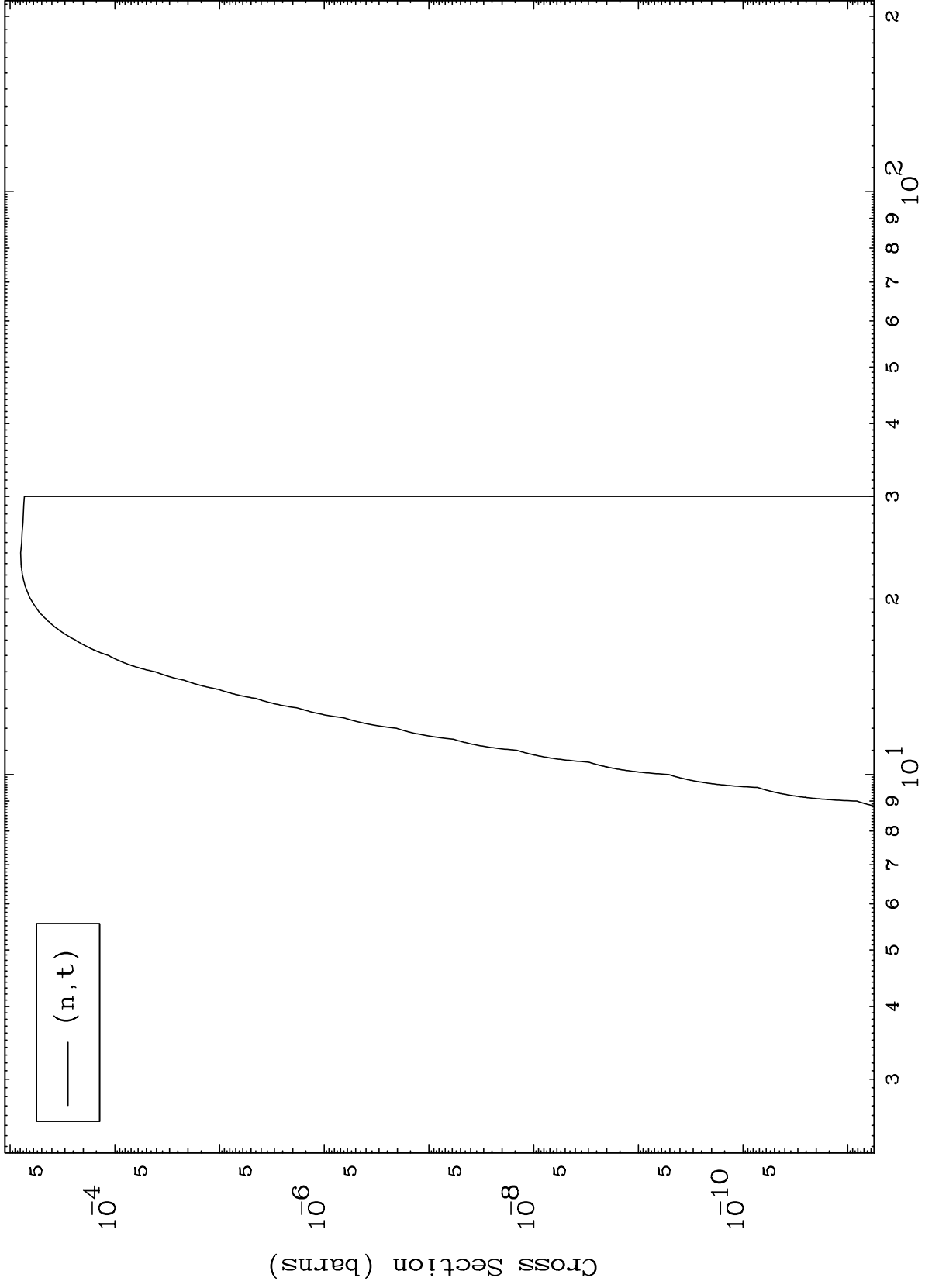
Incident Energy (MeV)

10

MAT 9938

(n,t) Levels
293 Kelvin Cross Sections

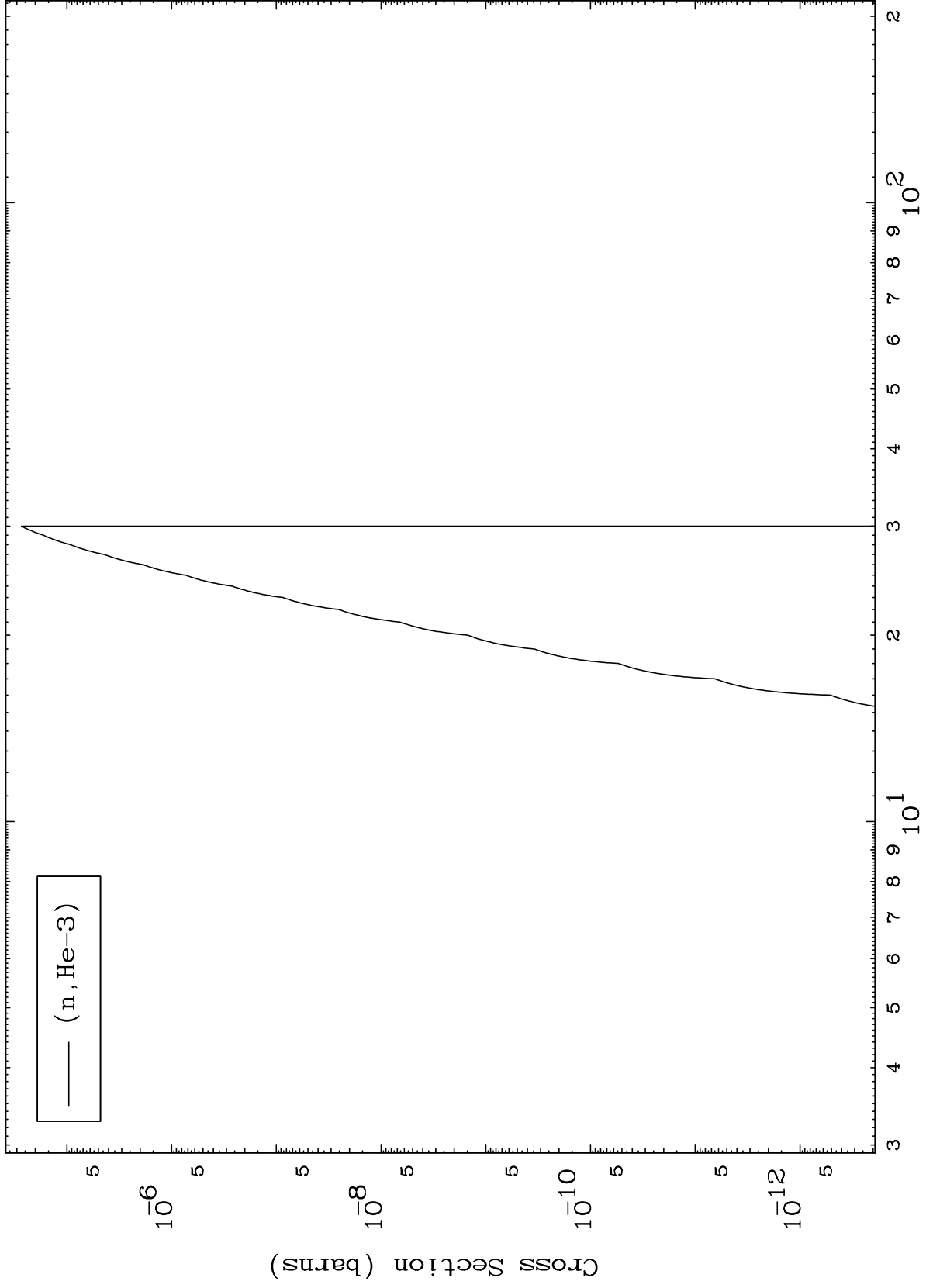
100-Fm-257



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(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-257



12

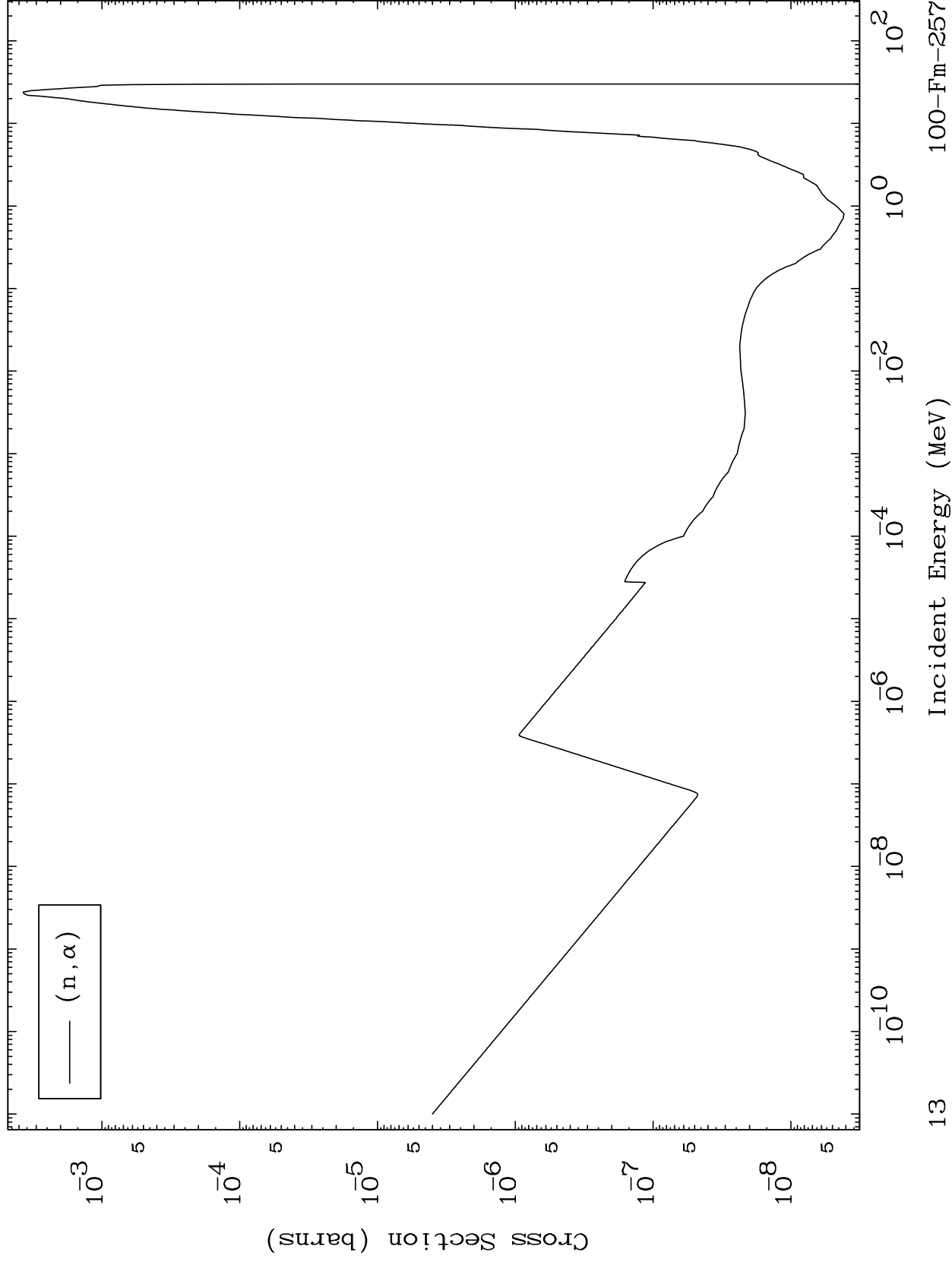
Incident Energy (MeV)

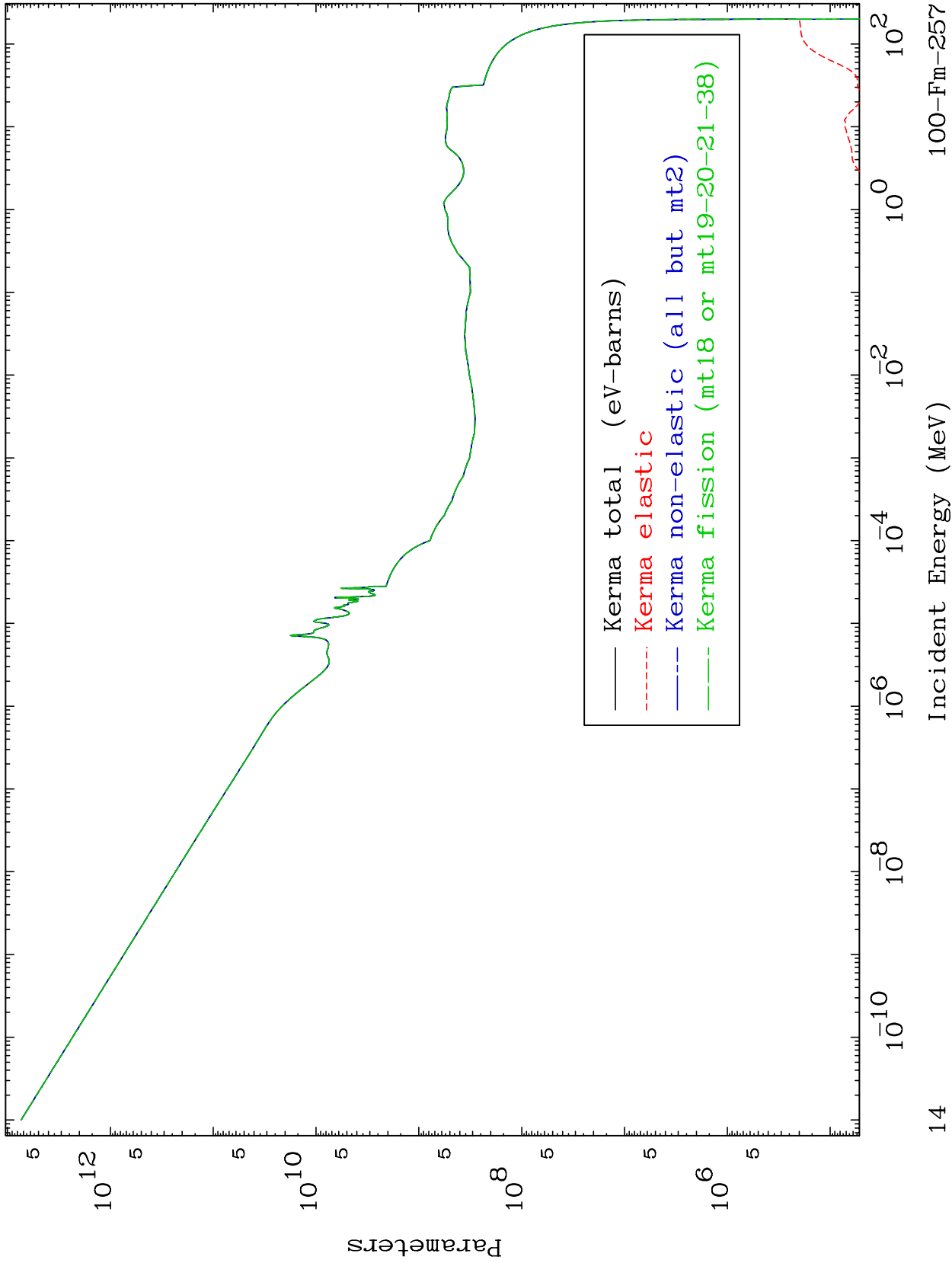
100-Fm-257

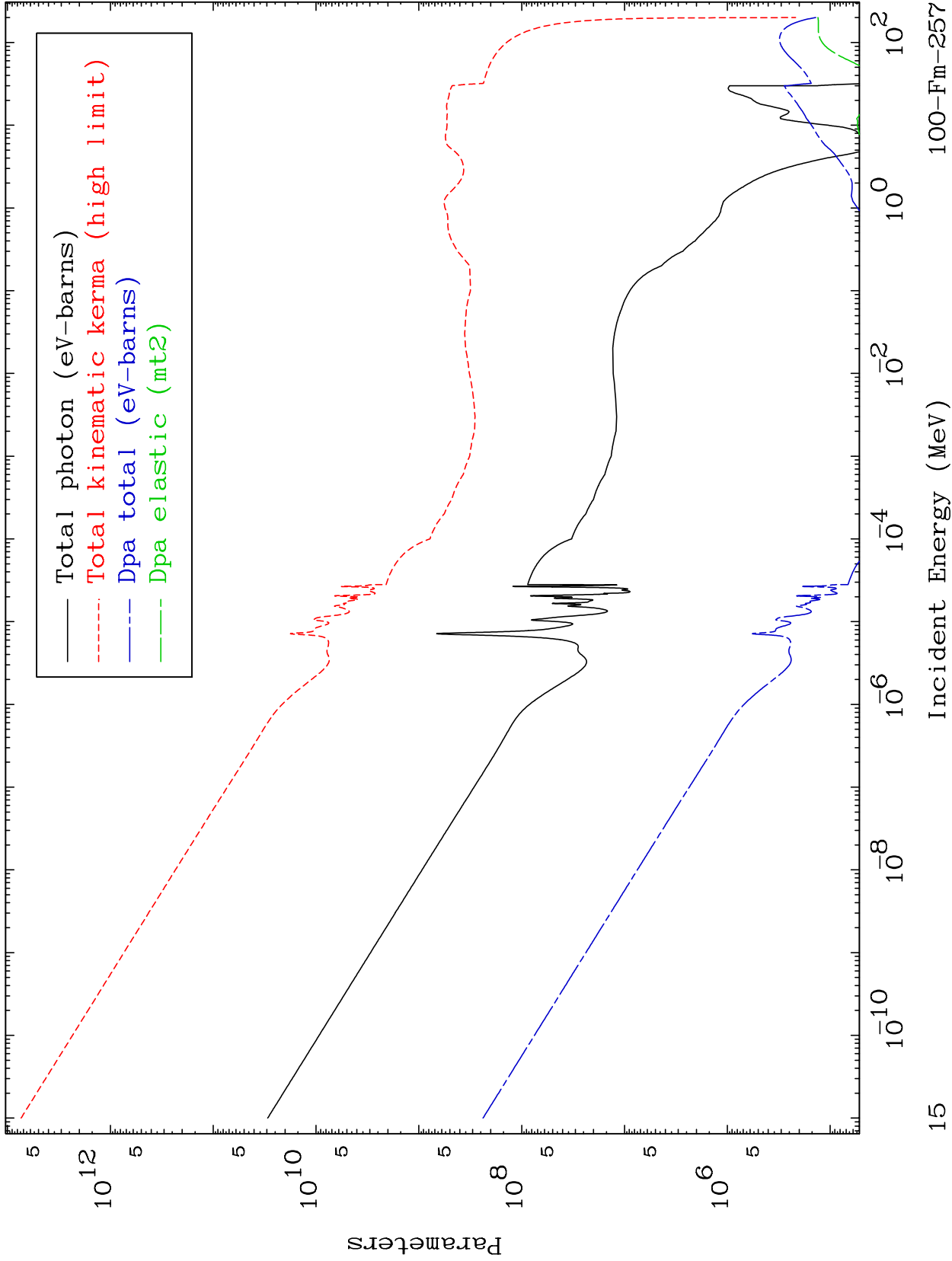
MAT 9938

(n, α) Levels
293 Kelvin Cross Sections

100-Fm-257



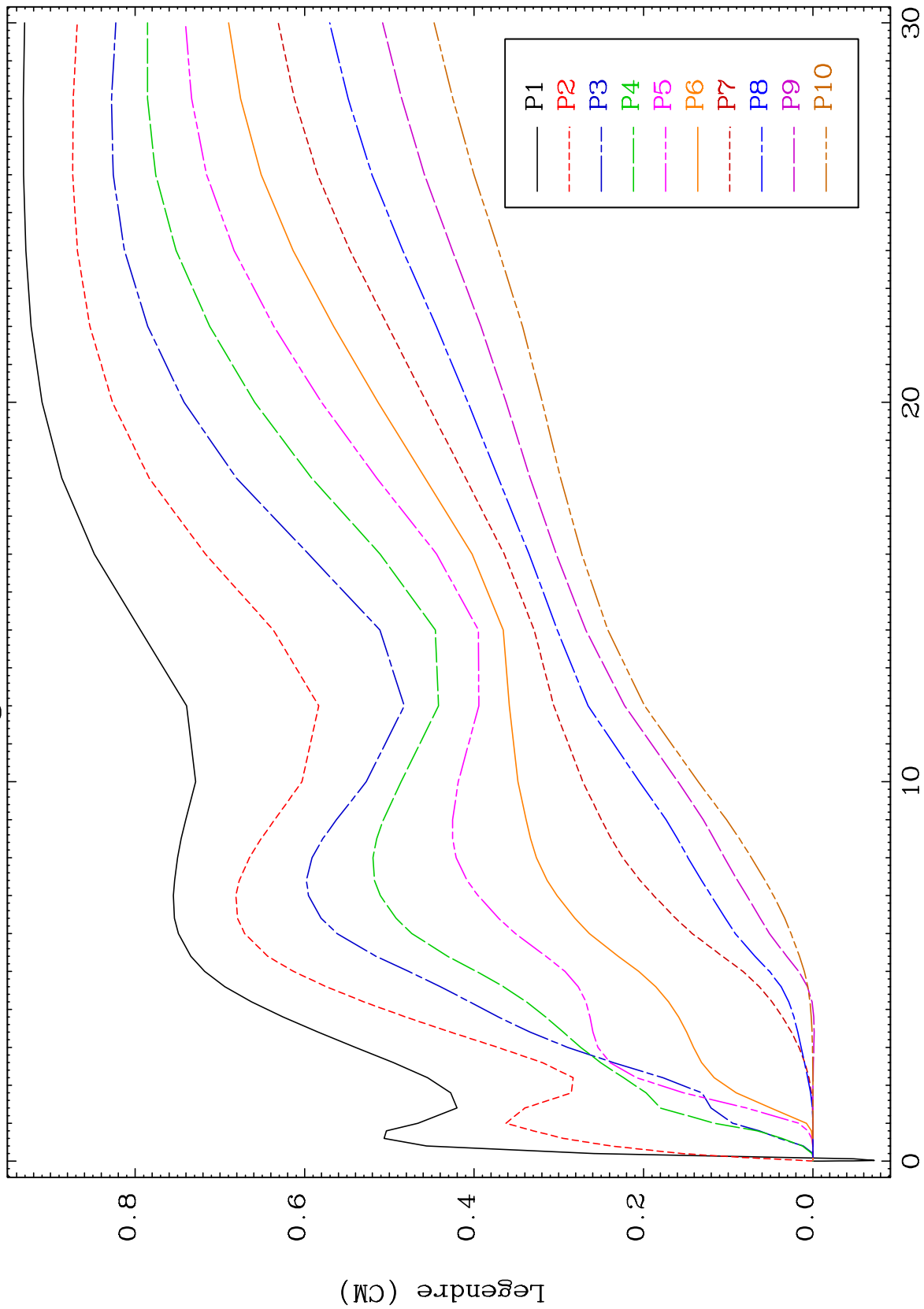




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Elastic Legendre Coefficients

100-Fm-257



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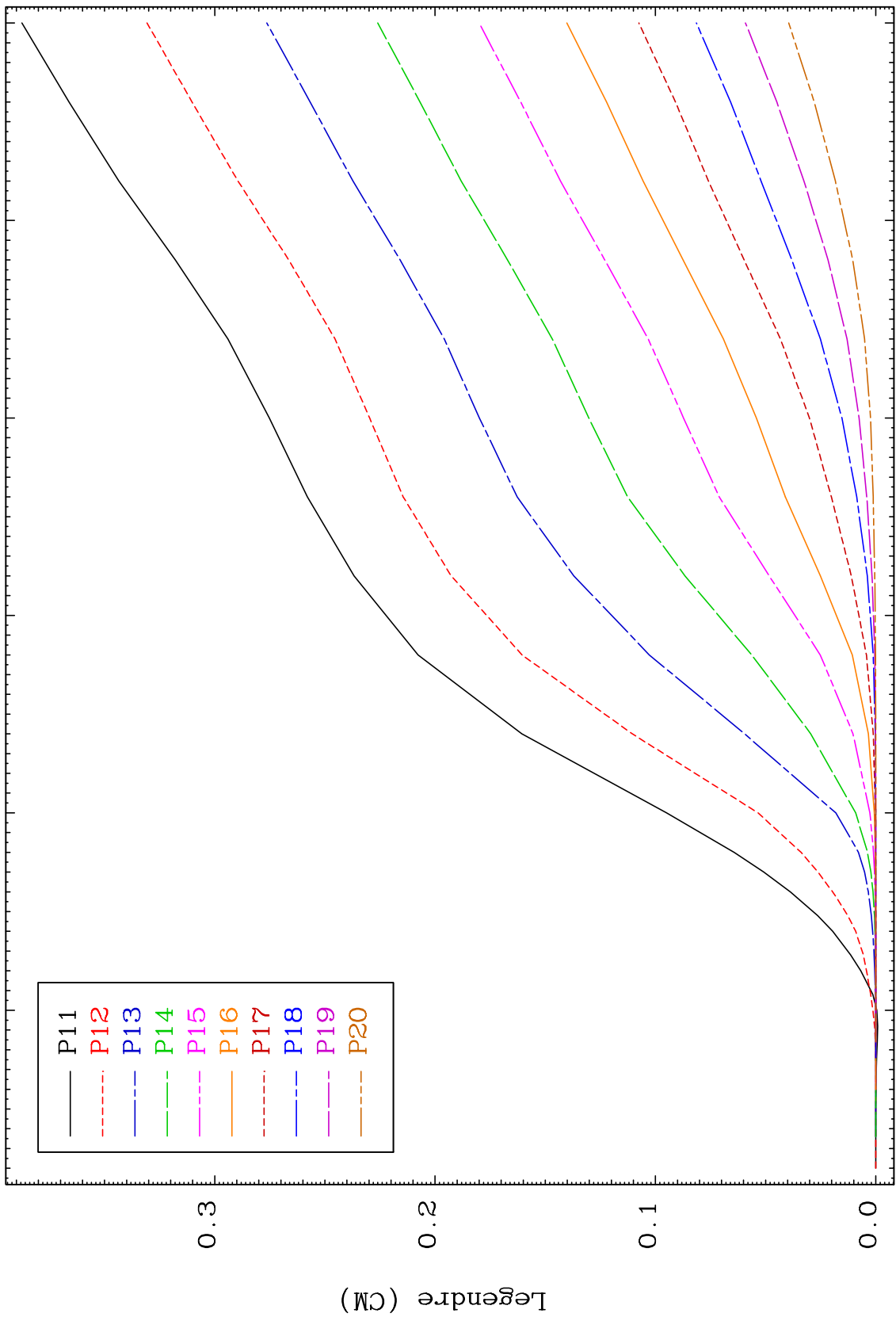
Incident Energy (MeV)

100-Fm-257

MAT 9938

Elastic Legendre Coefficients

100-Fm-257



17

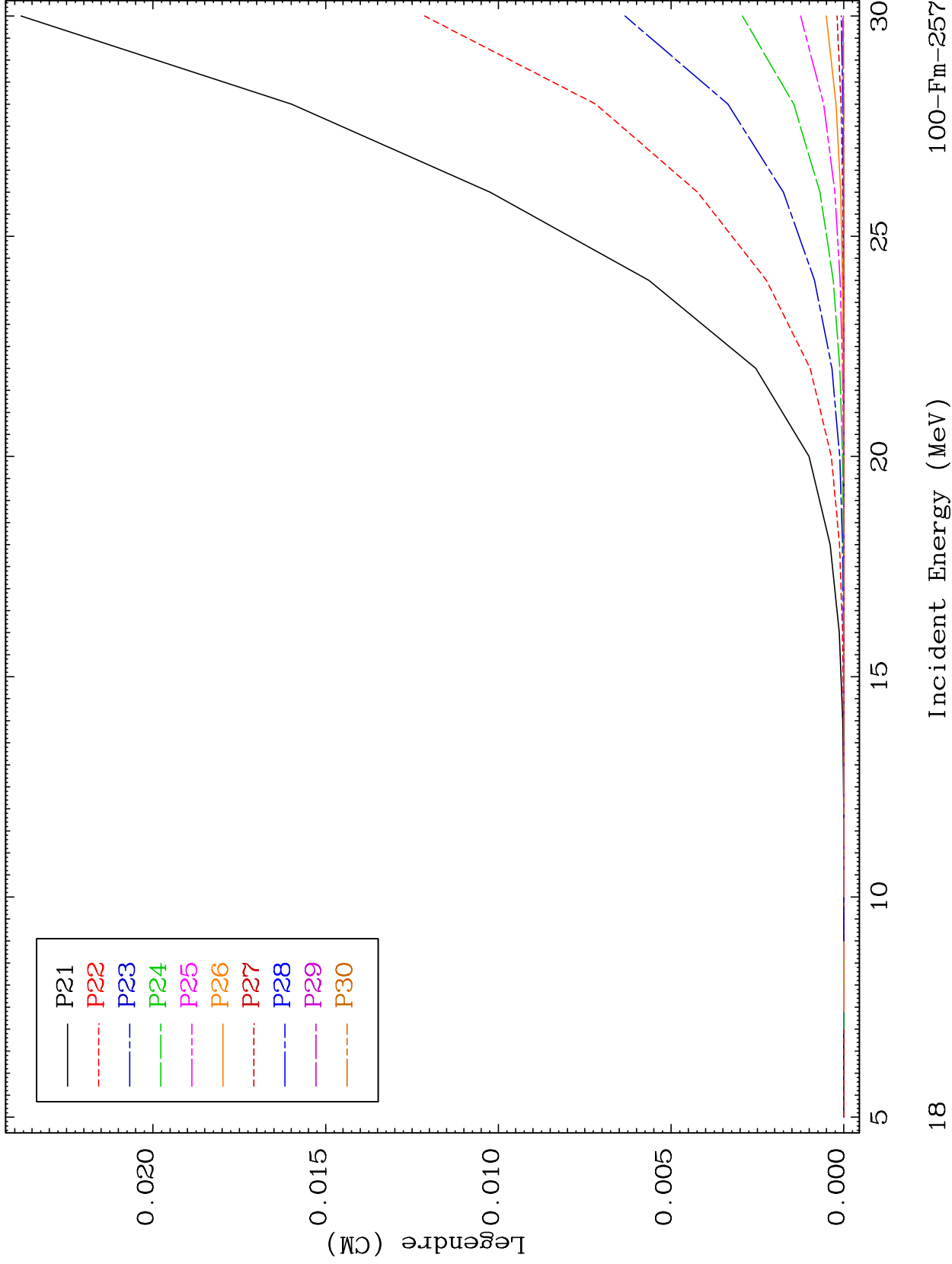
Incident Energy (MeV)

100-Fm-257

MAT 9938

Elastic Legendre Coefficients

100-Fm-257



18

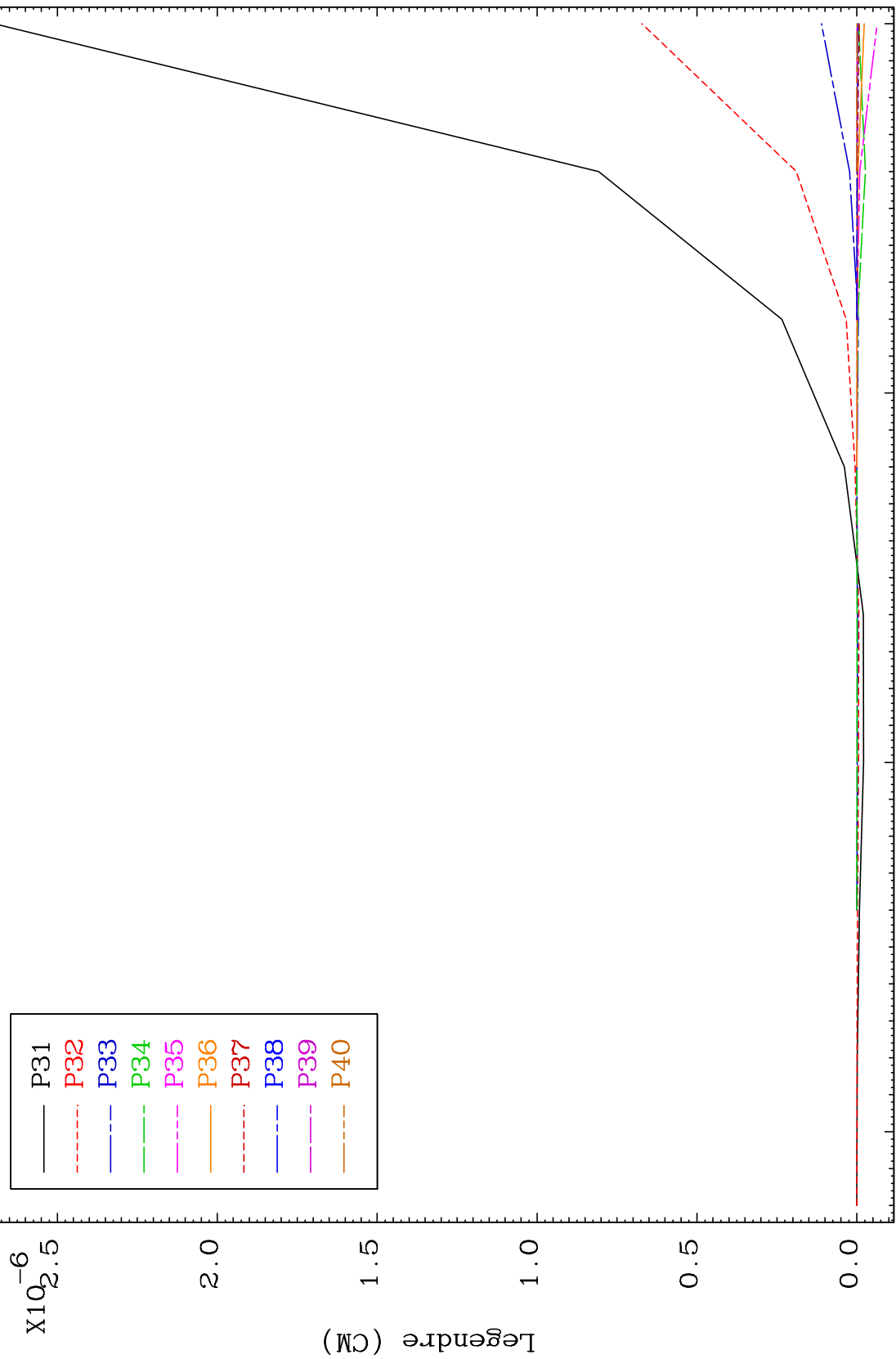
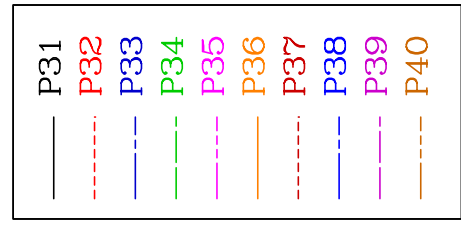
Incident Energy (MeV)

100-Fm-257

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Elastic Legendre Coefficients

100-Fm-257



19

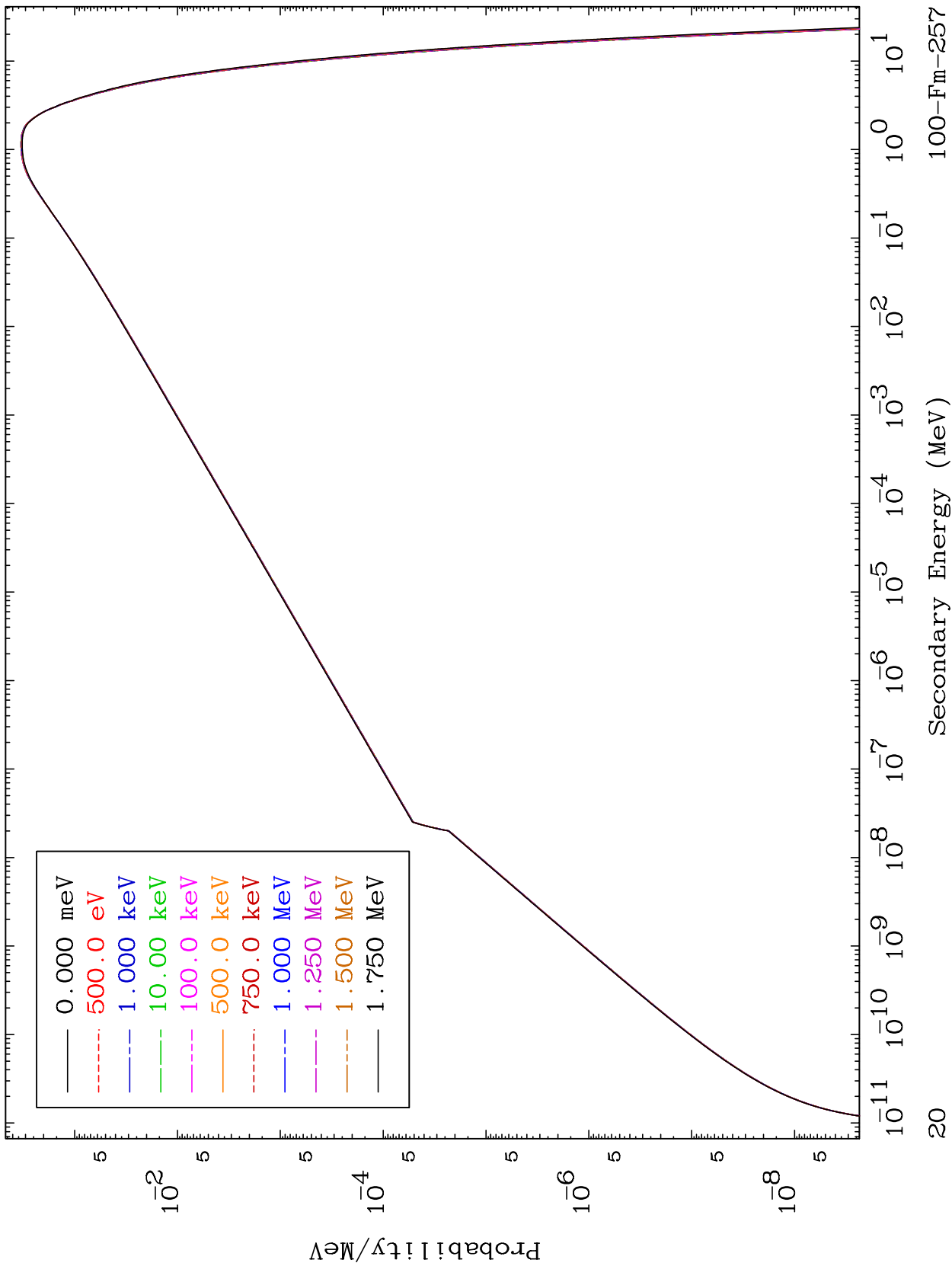
Incident Energy (MeV)

100-Fm-257

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Fission Energy Distribution

100-Fm-257



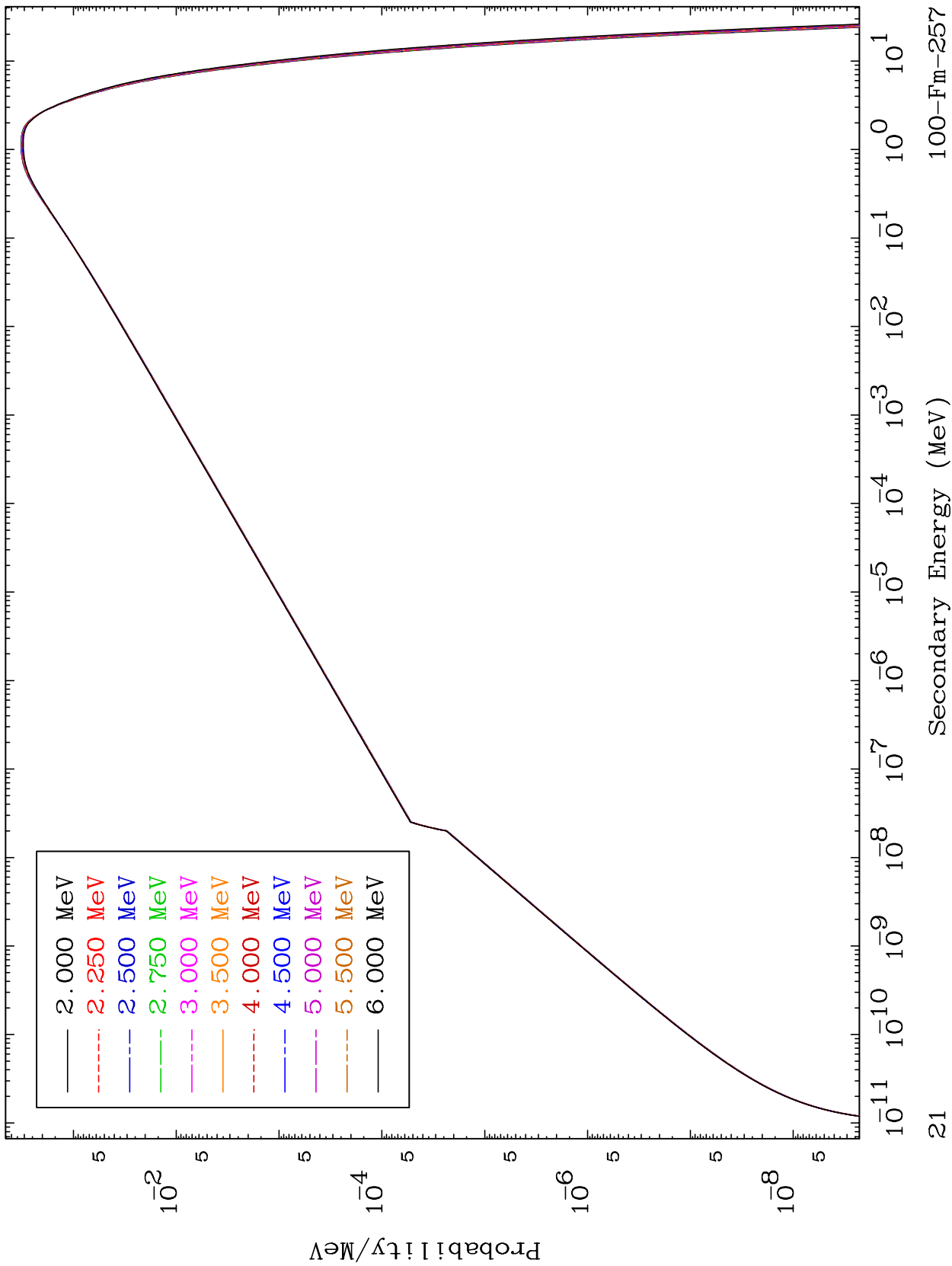
100-Fm-257

Secondary Energy (MeV)

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Fission Energy Distribution

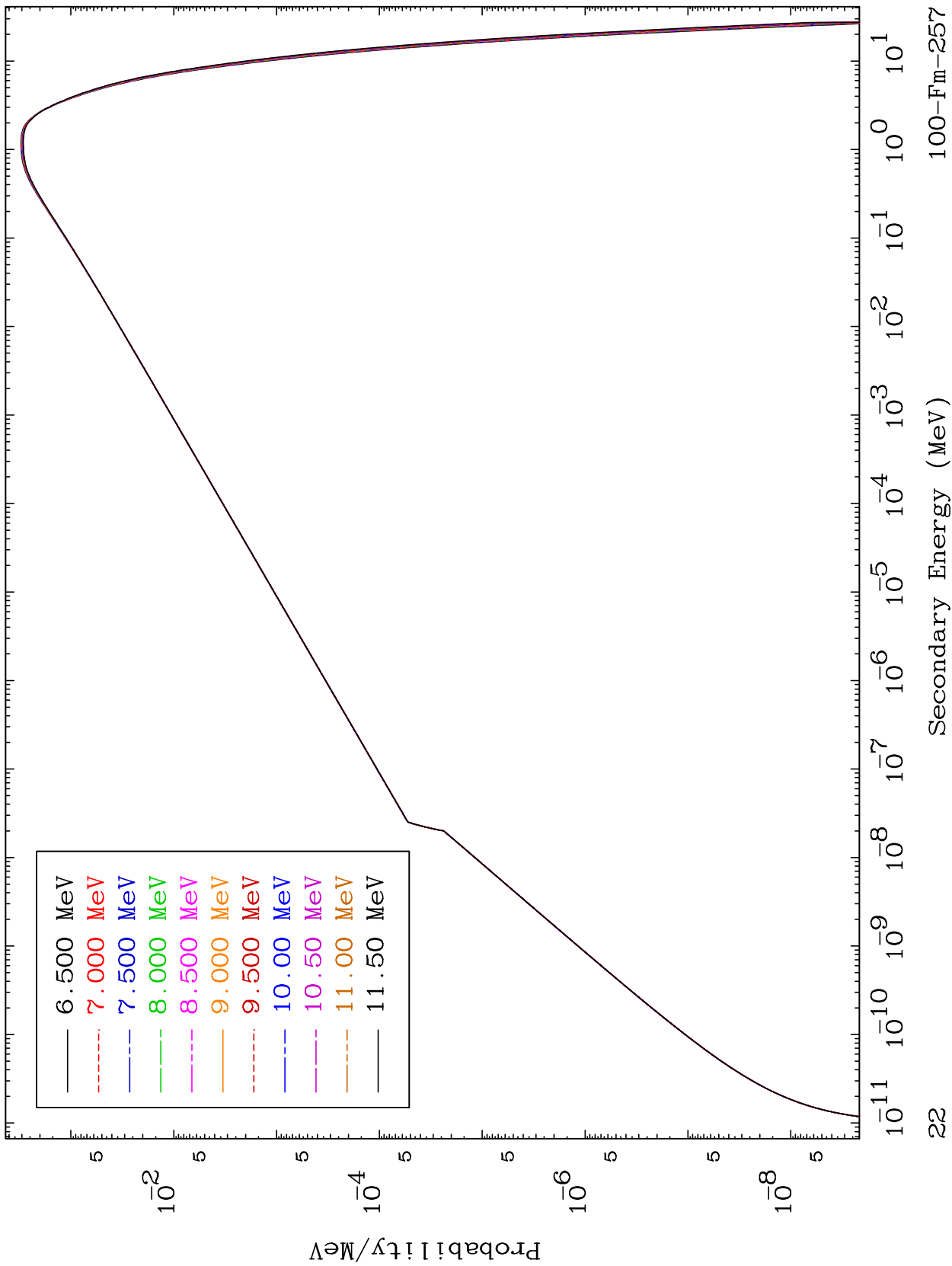
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Fission Energy Distribution

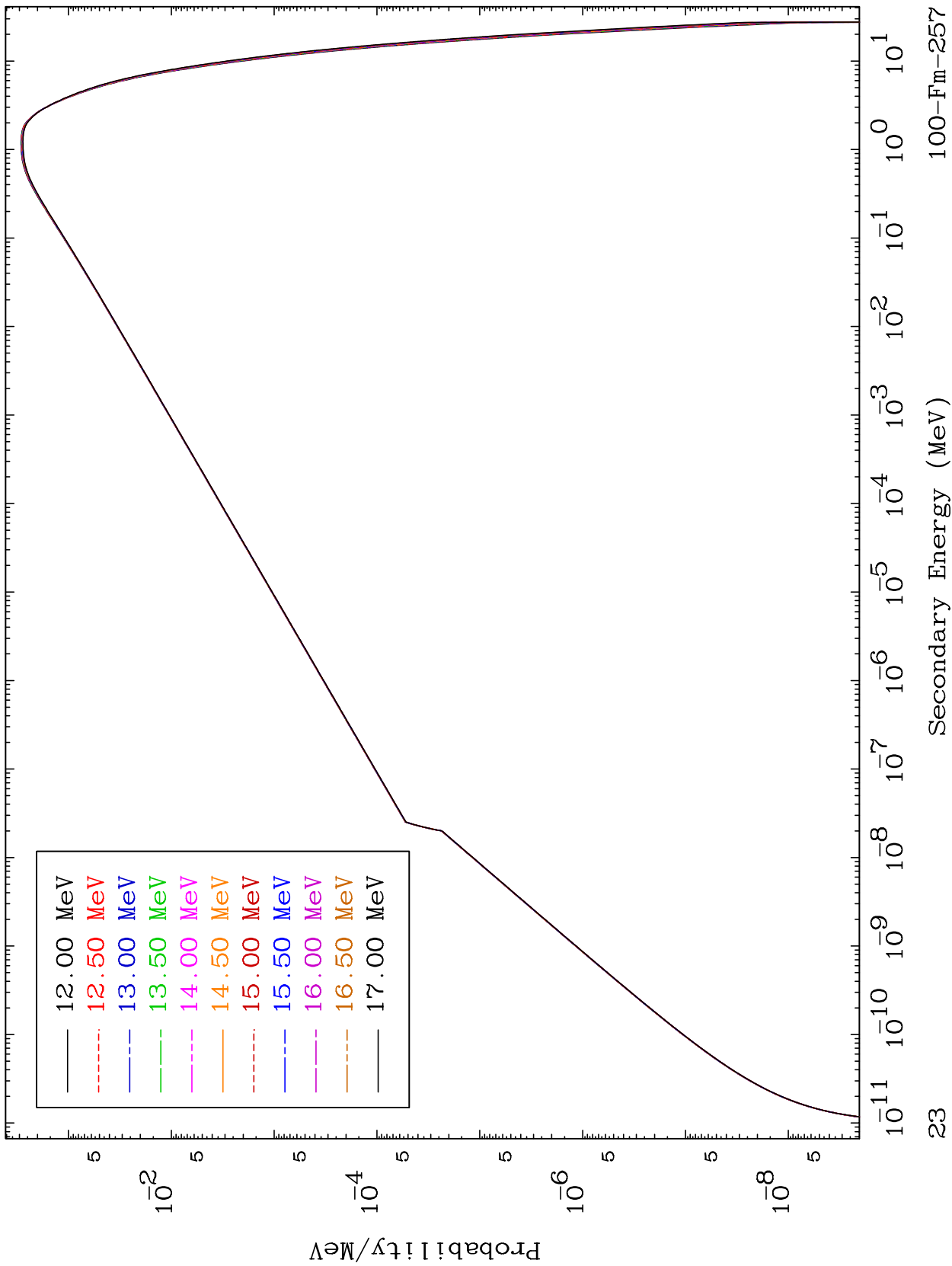
100-Fm-257



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Fission Energy Distribution

100-Fm-257



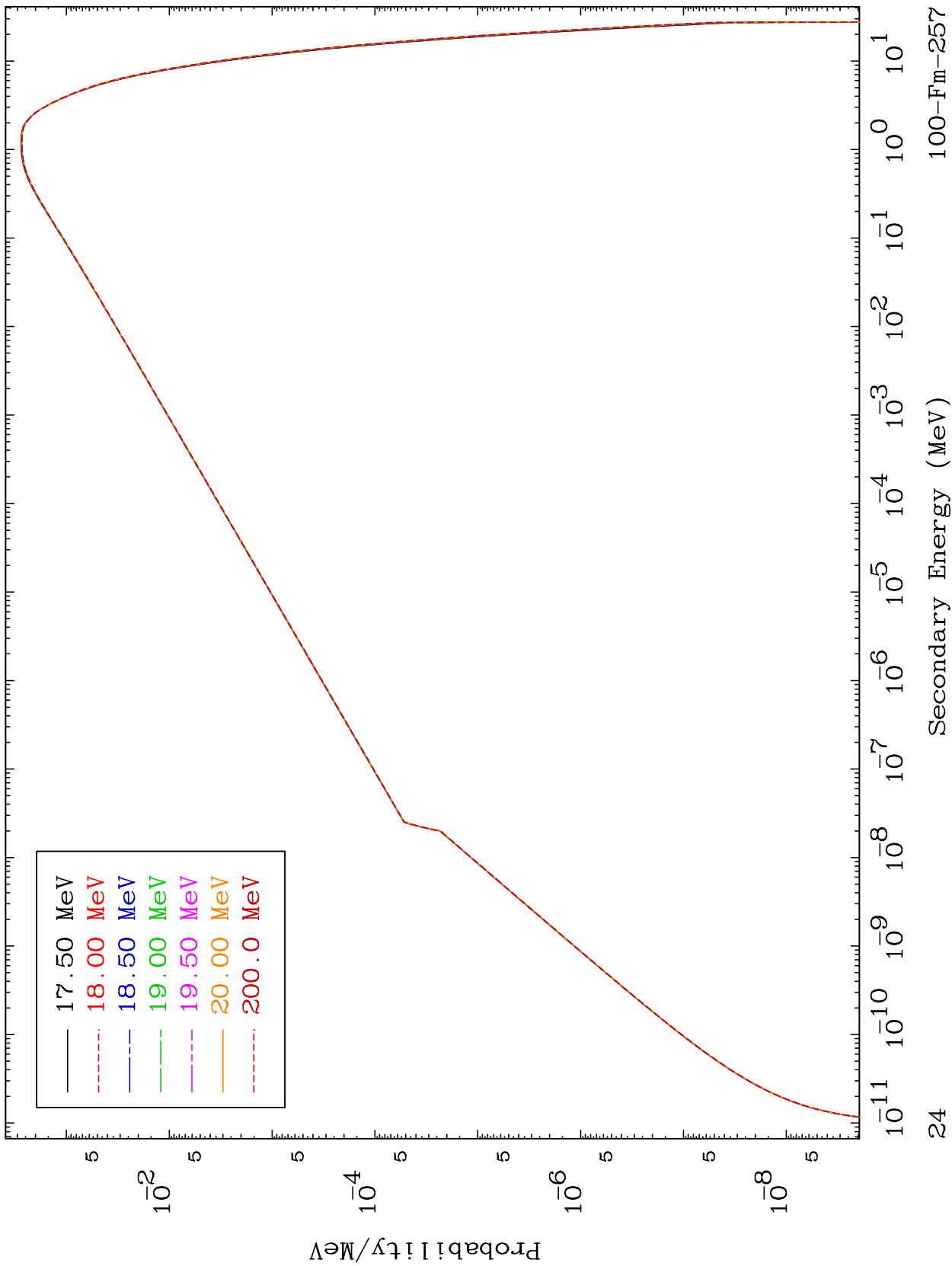
100-Fm-257

Secondary Energy (MeV)

MAT 9938

Fission Energy Distribution

100-Fm-257

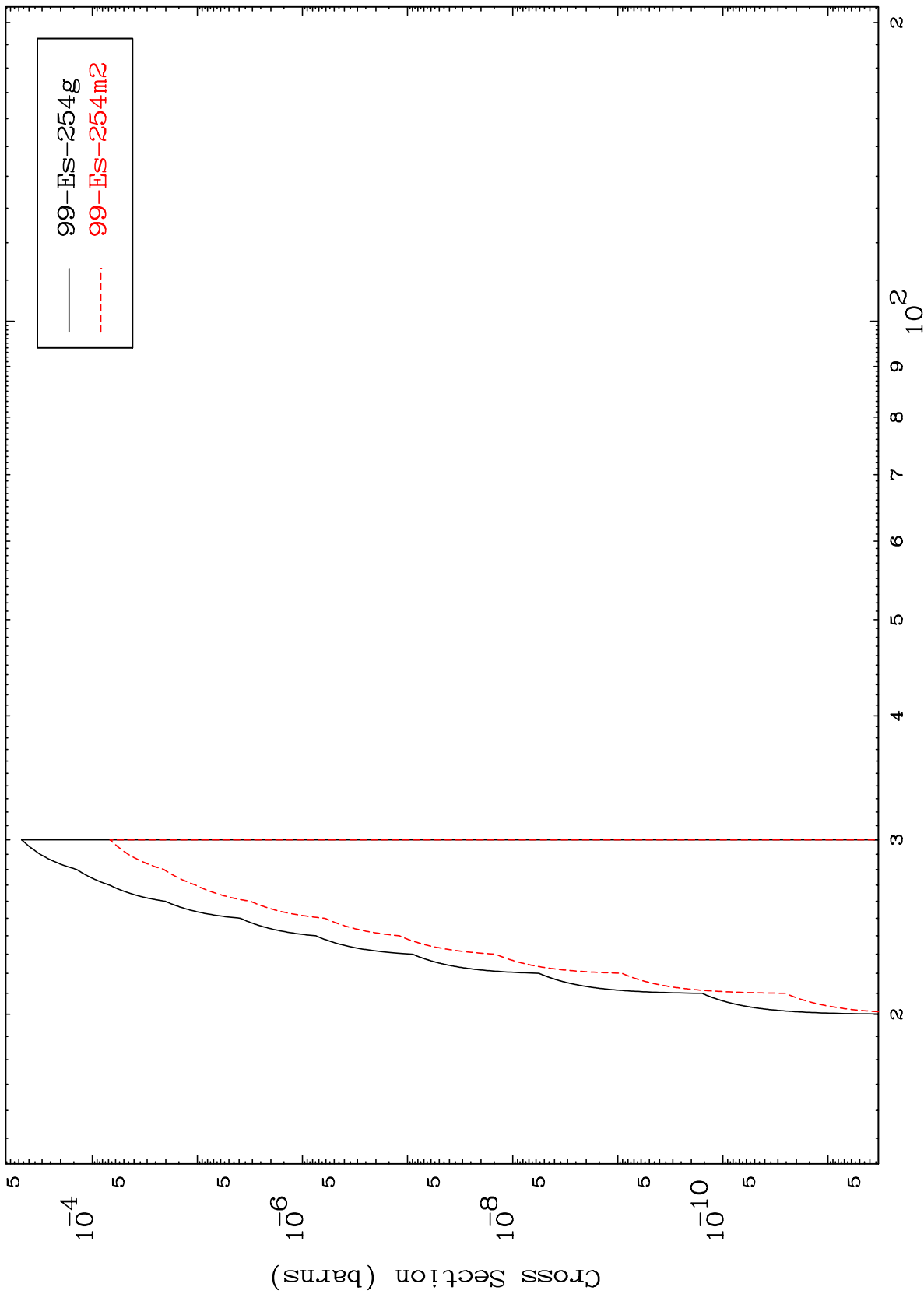


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(n,2n) d

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Radionuclide Production Cross Section



25

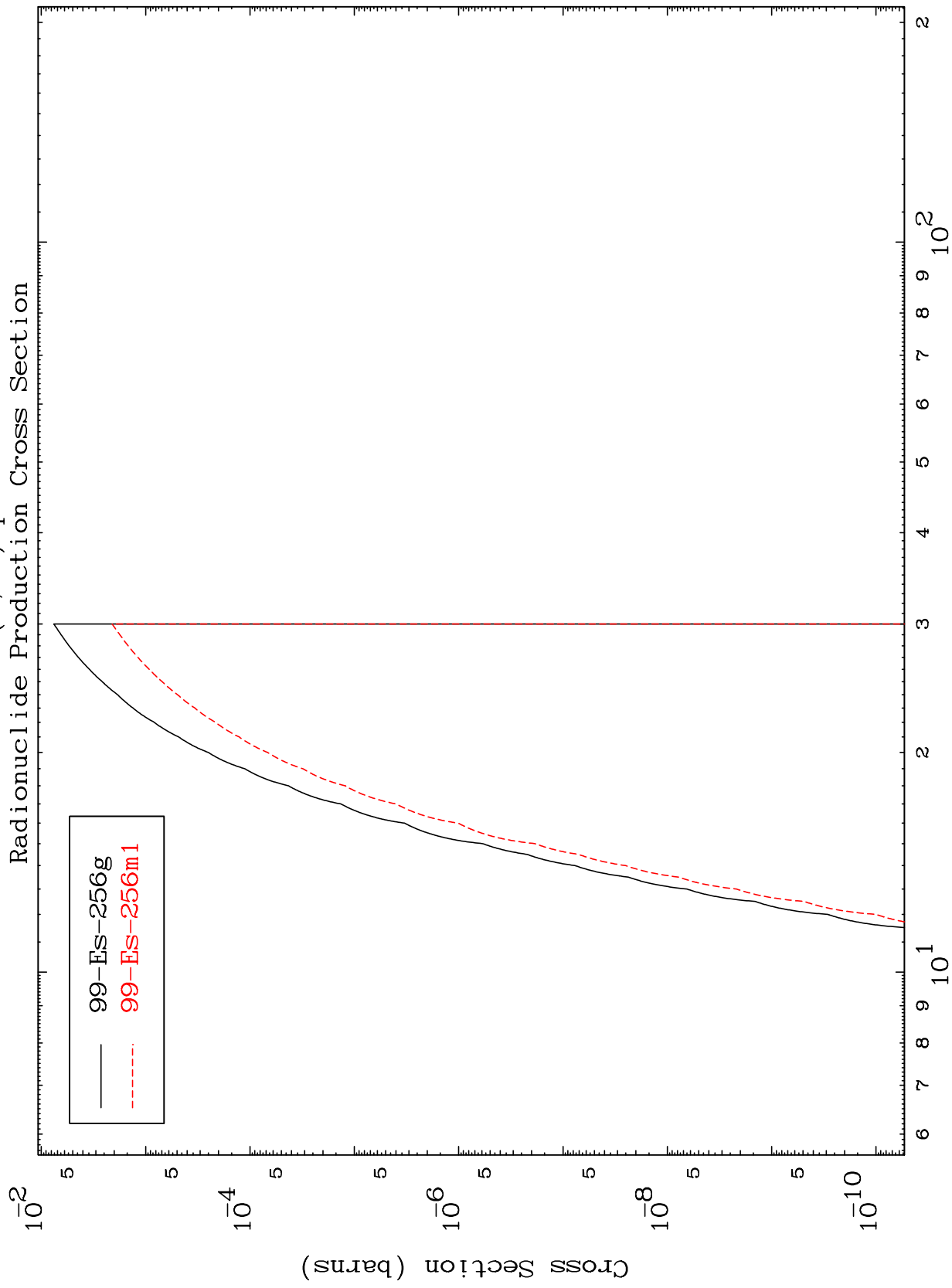
Incident Energy (MeV)

100-Fm-257

MAT 9938

(n,n') p

100-Fm-257



26

Incident Energy (MeV)

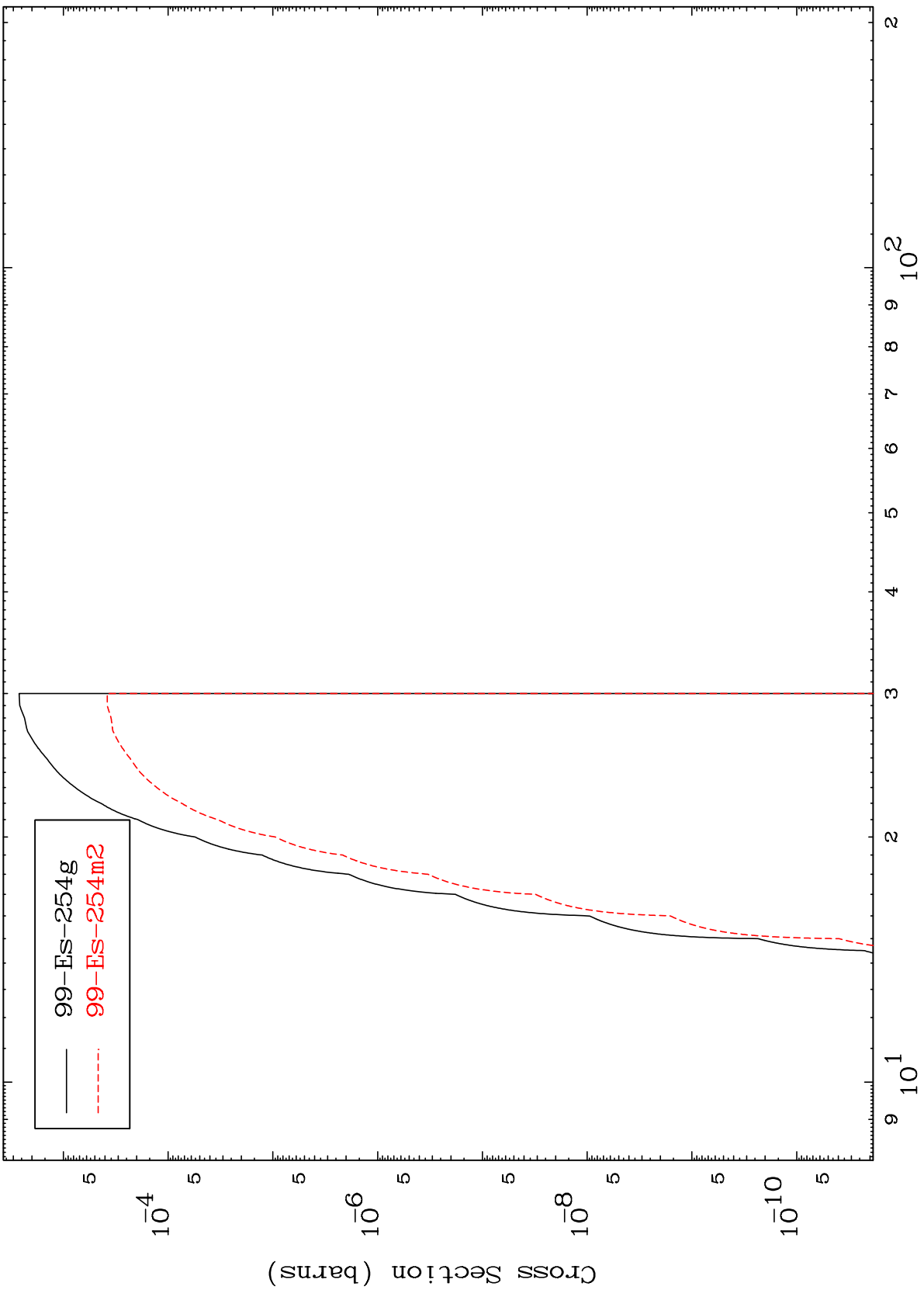
100-Fm-257

MAT 9938

(n,n') t

100-Fm-257

Radionuclide Production Cross Section



27

Incident Energy (MeV)

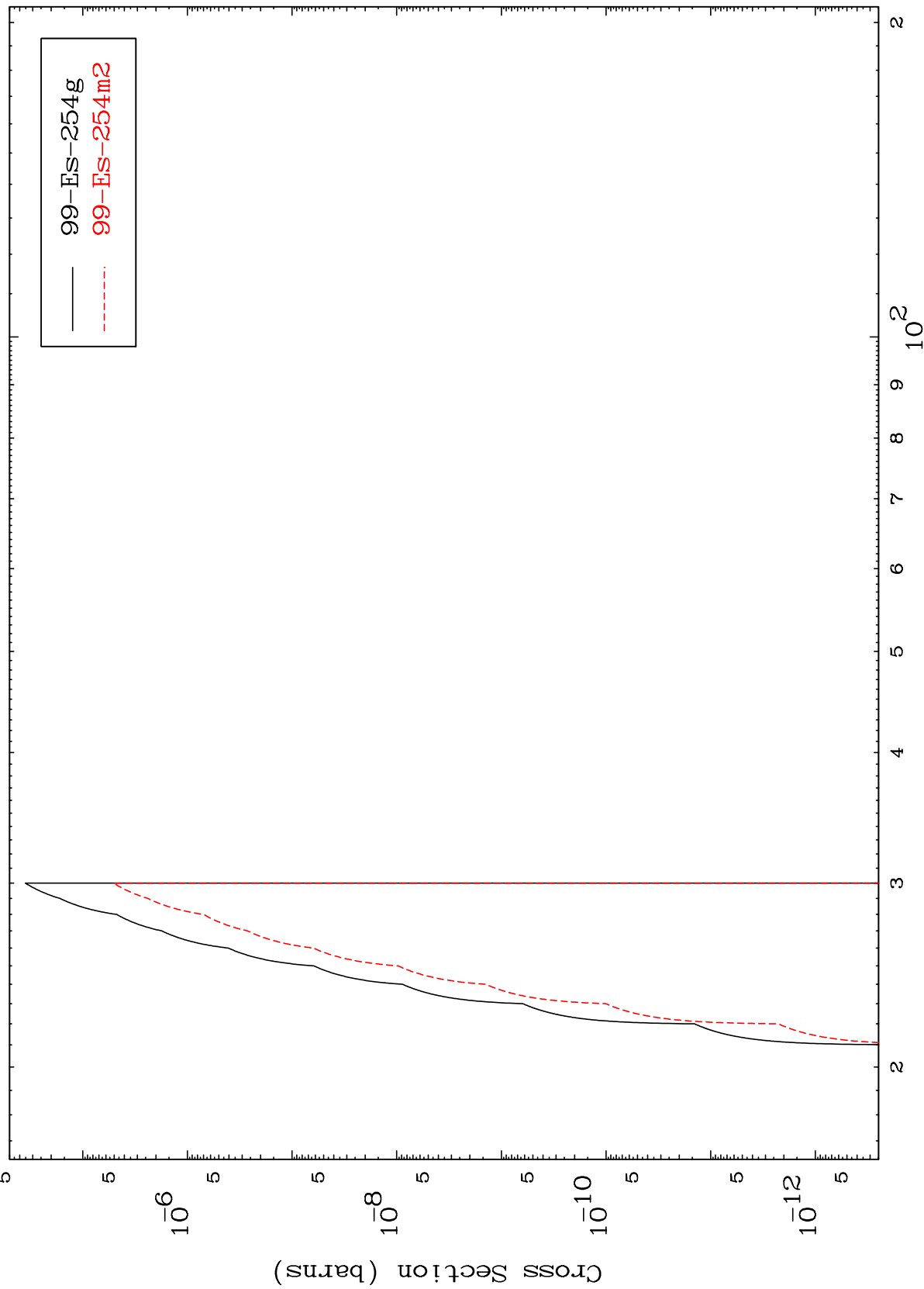
100-Fm-257

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(n,3n) p

100-Fm-257

Radionuclide Production Cross Section



28

Incident Energy (MeV)

100-Fm-257

