

Program EVALPLOT  
(Version 2018-1)

by

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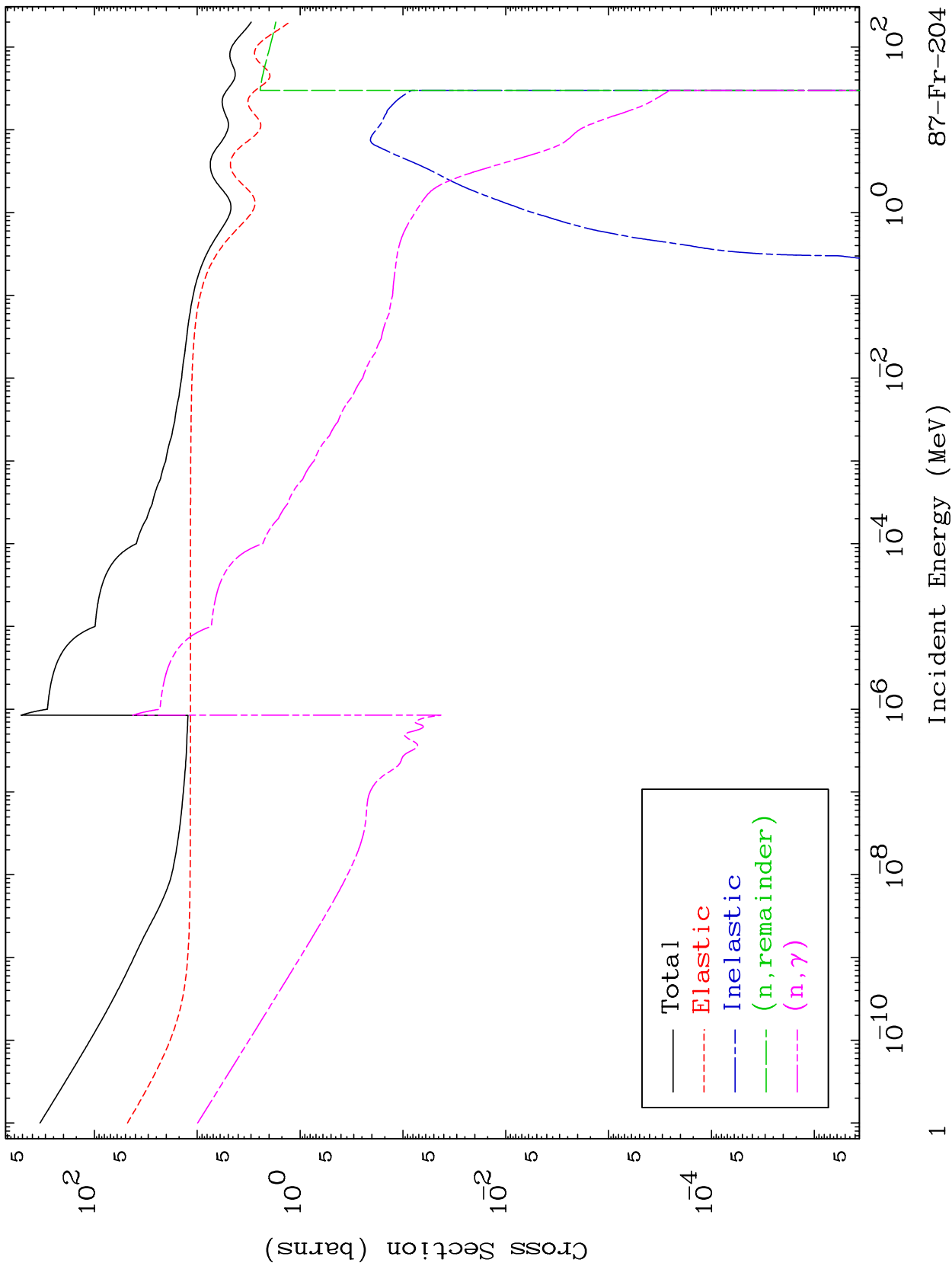
Press Mouse Button to Start

MAT 8702

Major

293 Kelvin Cross Sections

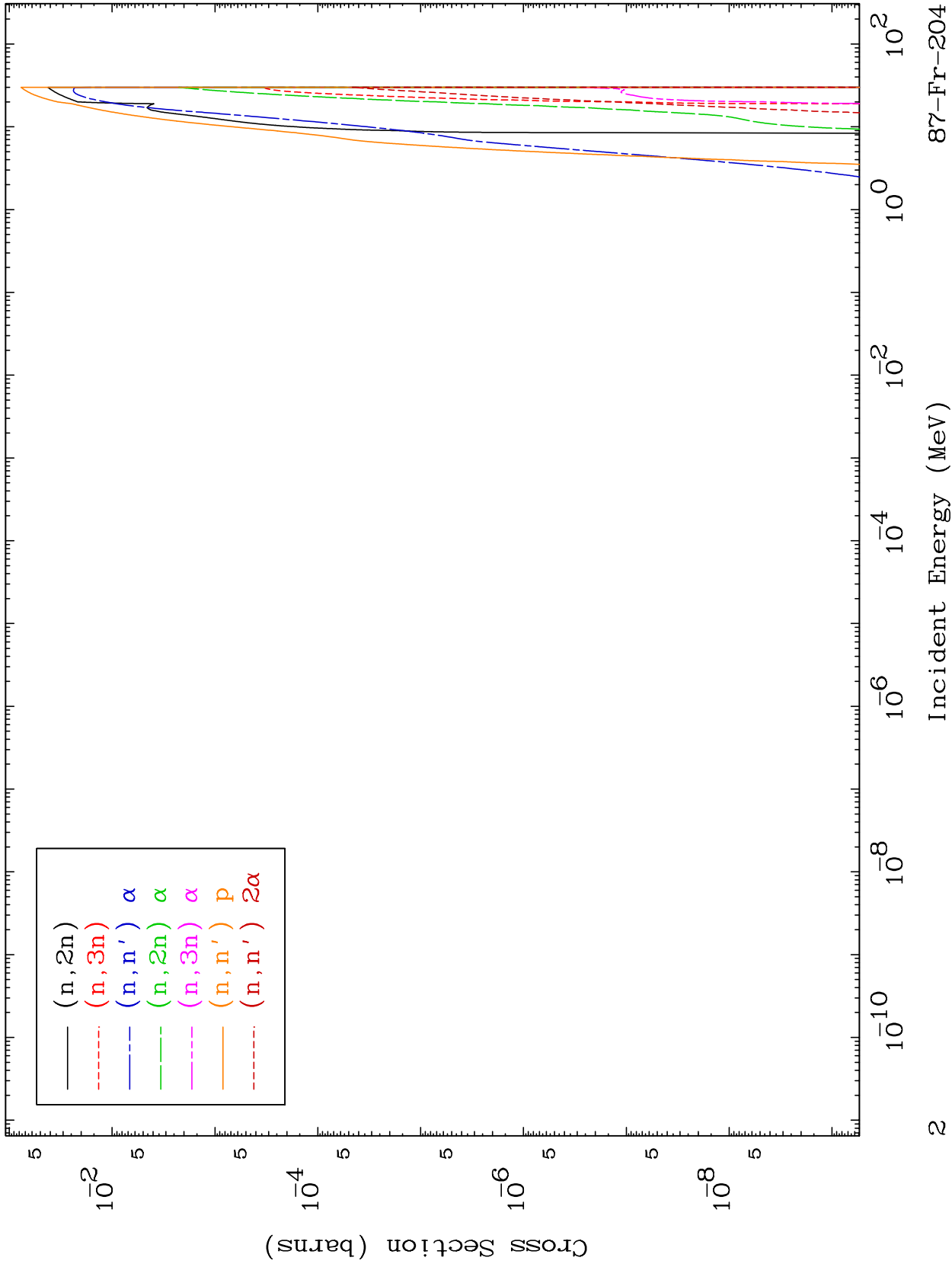
87-Fr-204



MAT 8702

Neutron Production  
293 Kelvin Cross Sections

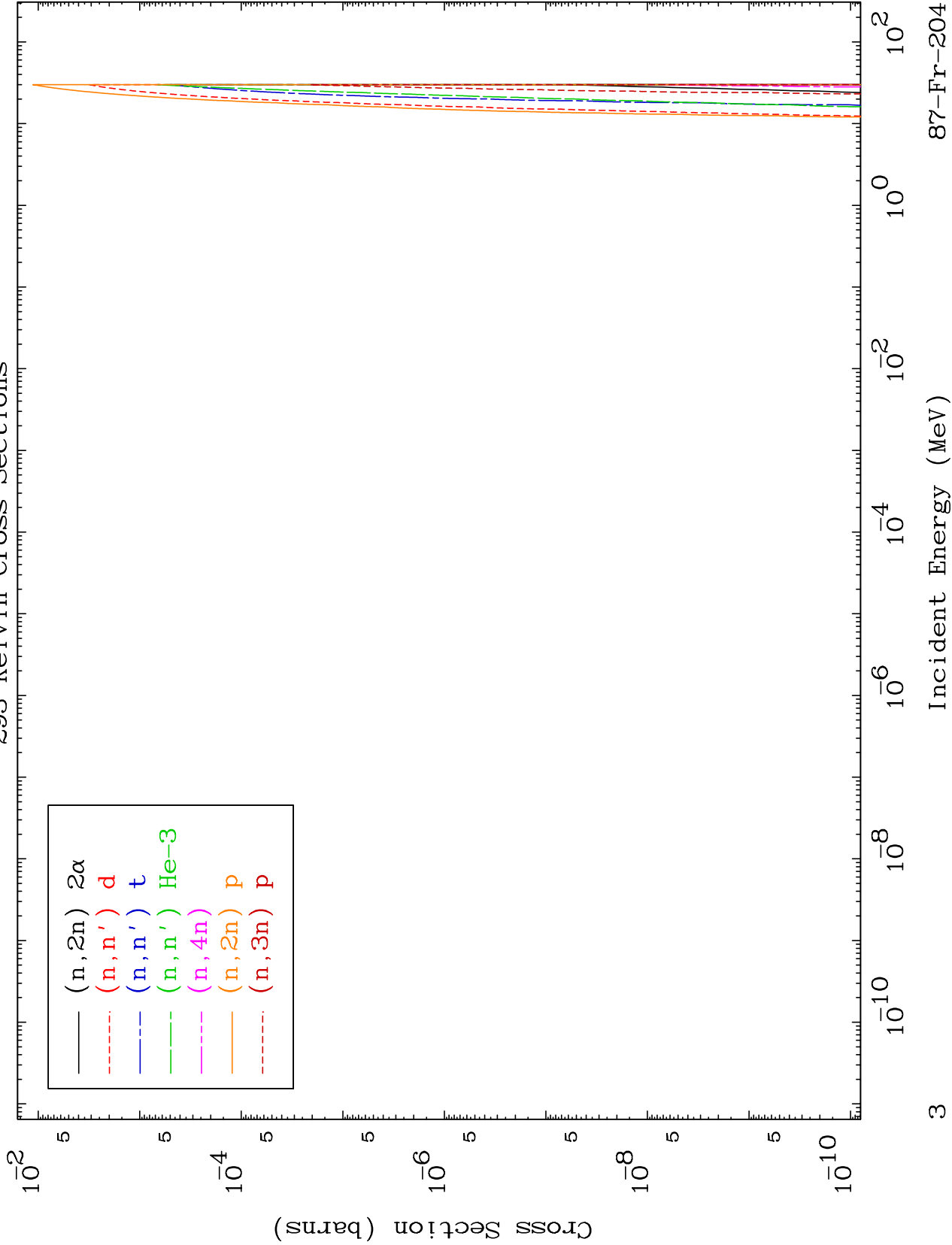
87-Fr-204



MAT 8702

Neutron Production  
293 Kelvin Cross Sections

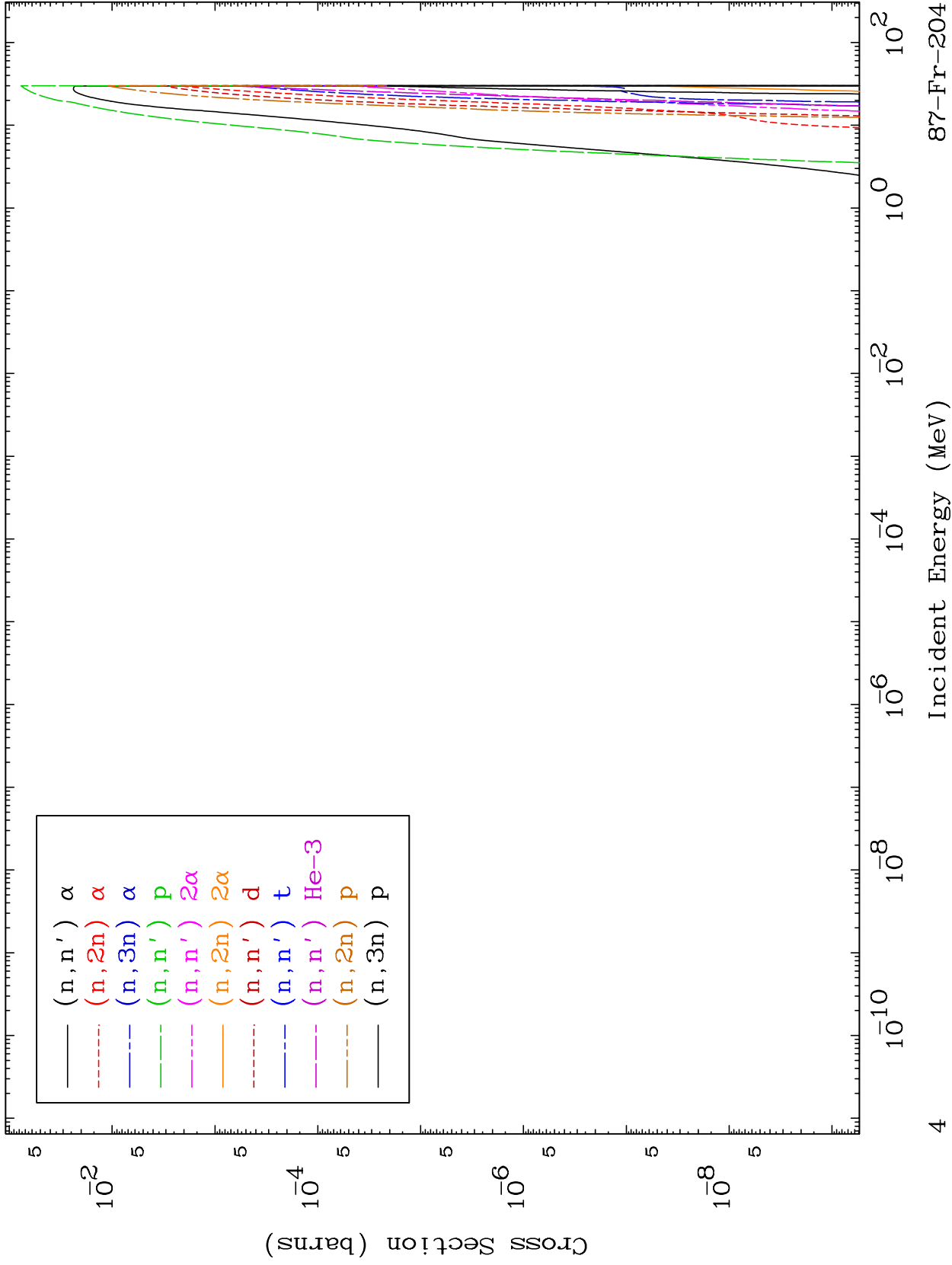
87-Fr-204

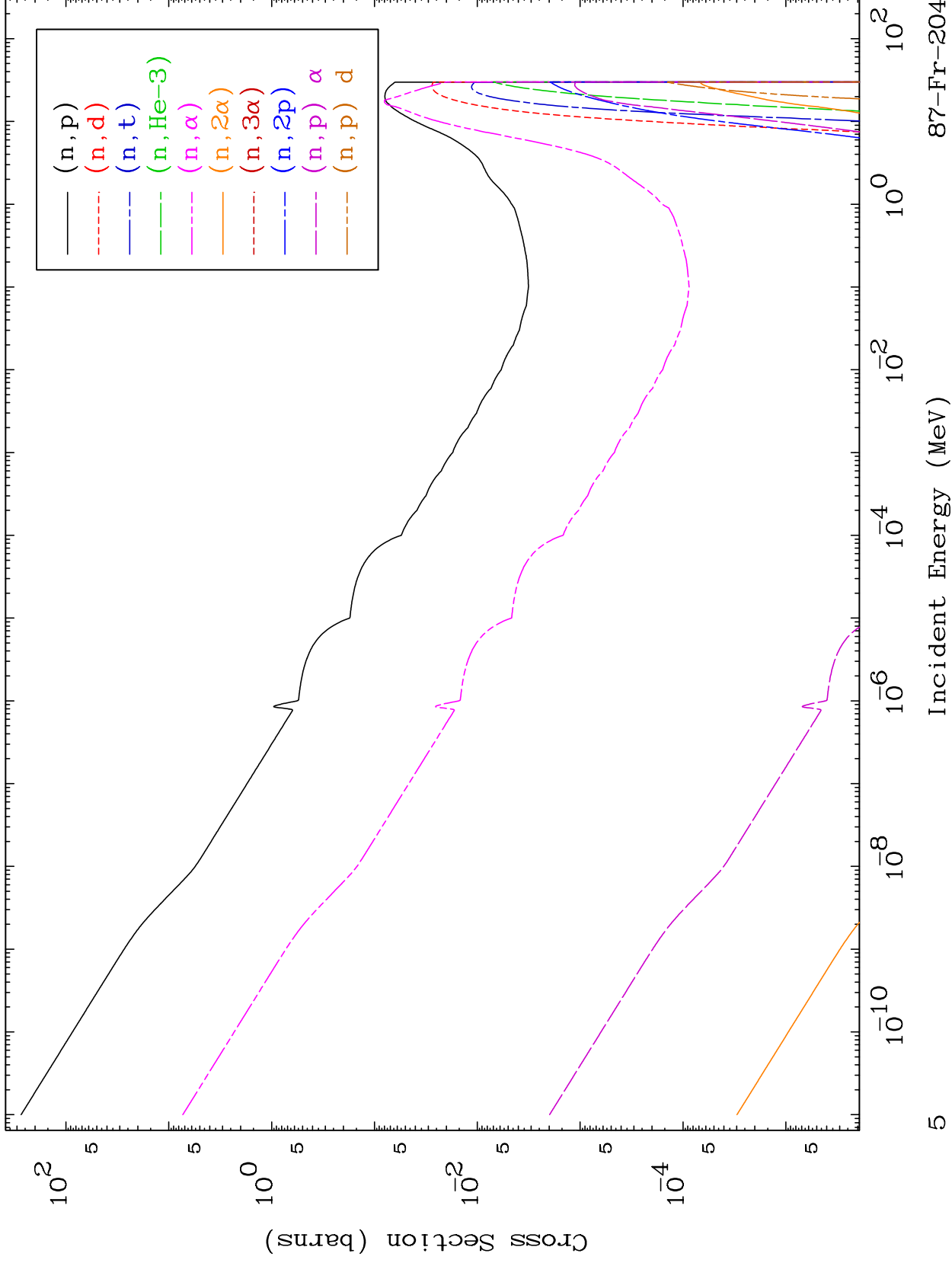


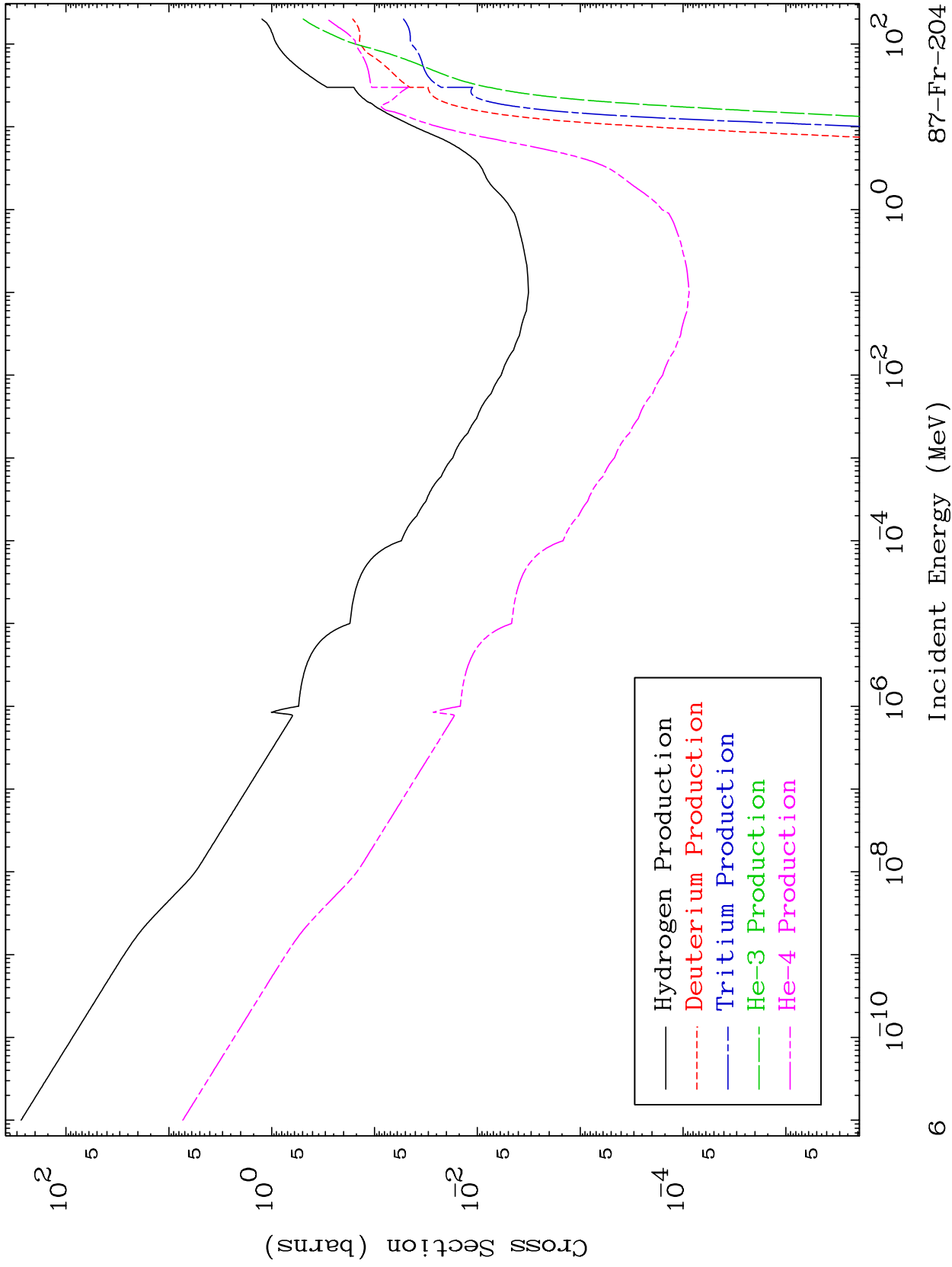
MAT 8702

Charged Particle  
293 Kelvin Cross Sections

87-Fr-204





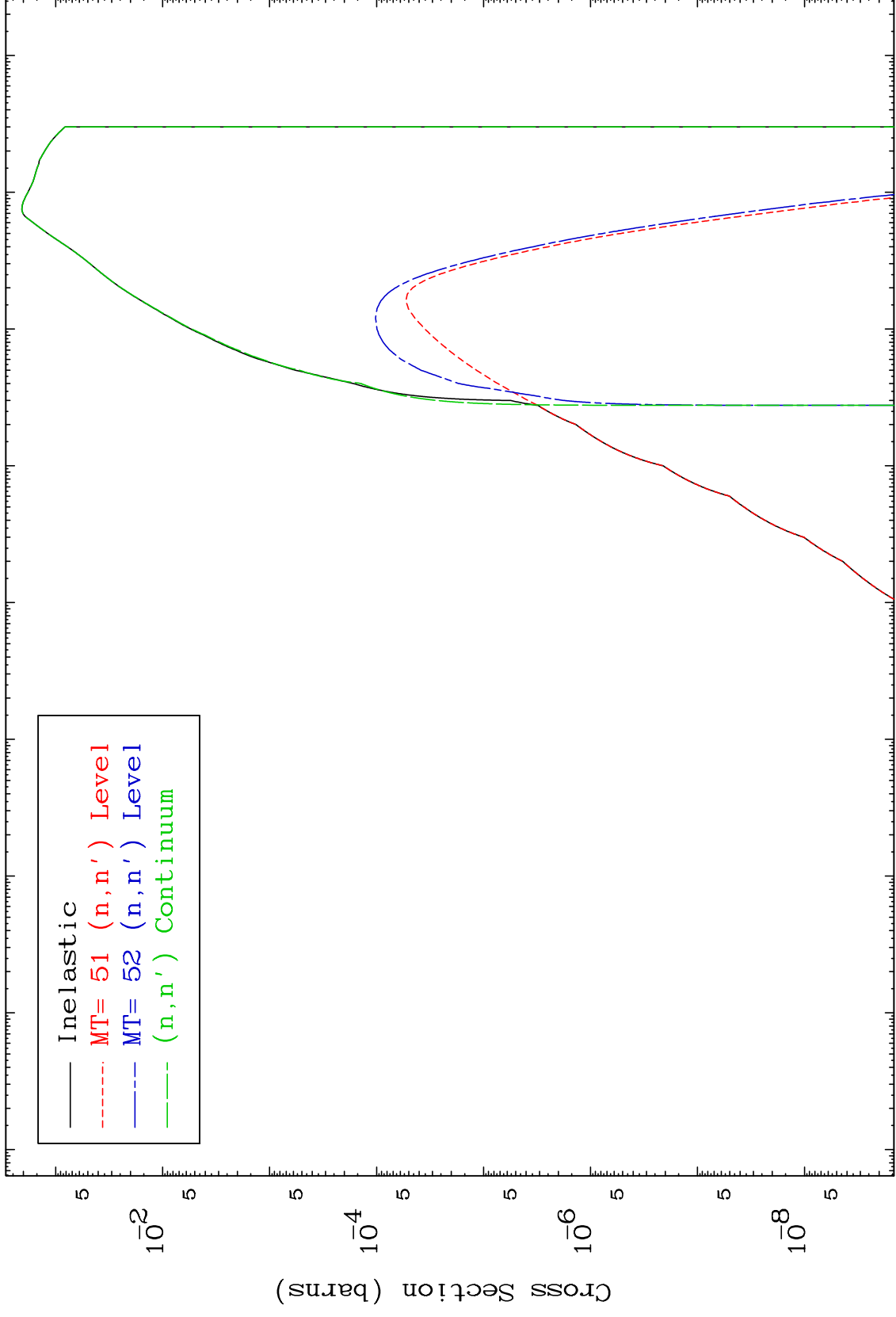


MAT 8702

(n,n') Level

87-Fr-204

293 Kelvin Cross Sections



Inelastic  
MT= 51 (n,n') Level  
MT= 52 (n,n') Level  
(n,n') Continuum

7

Incident Energy (MeV)

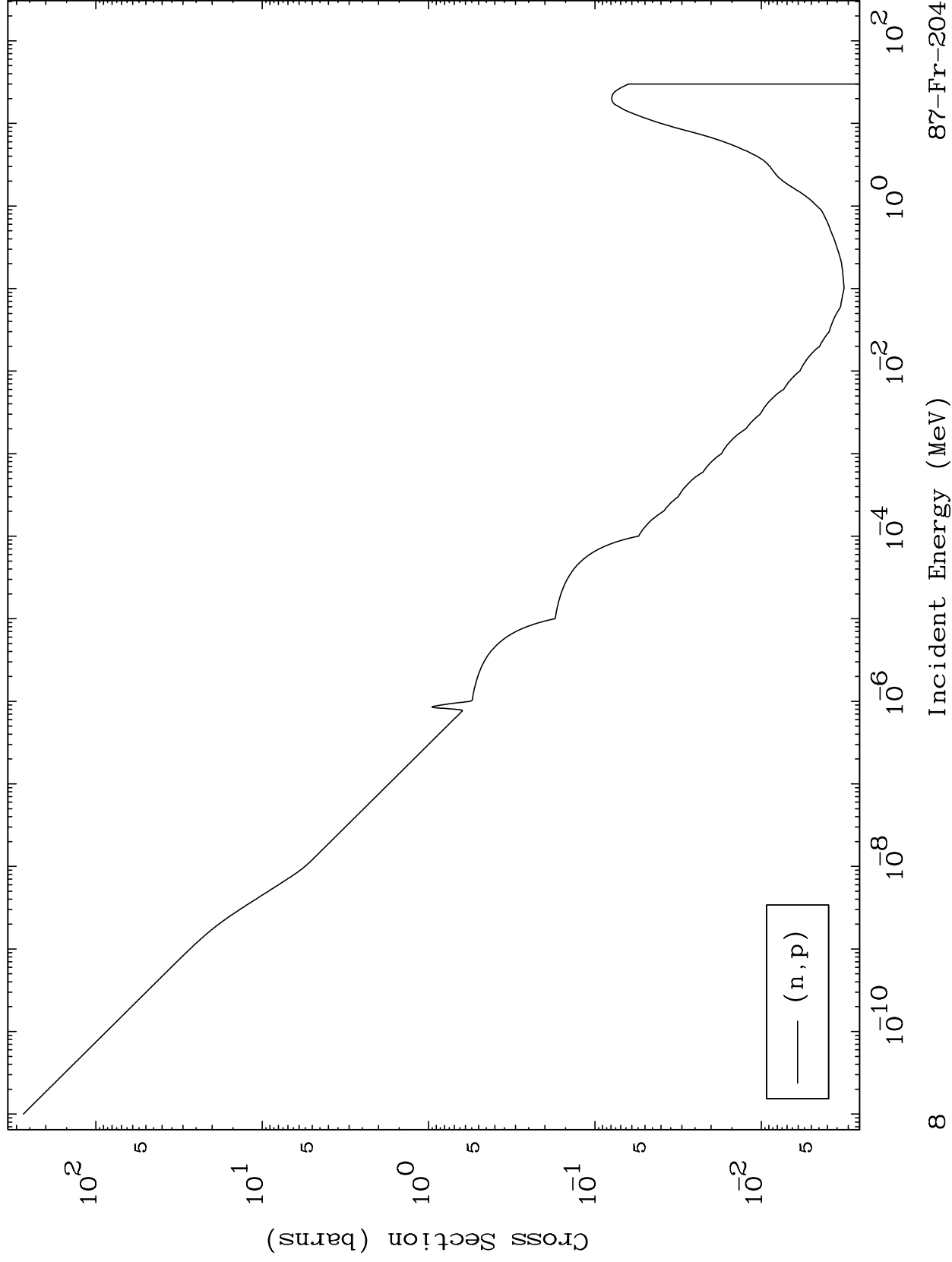
87-Fr-204



MAT 8702

(n,p) Levels  
293 Kelvin Cross Sections

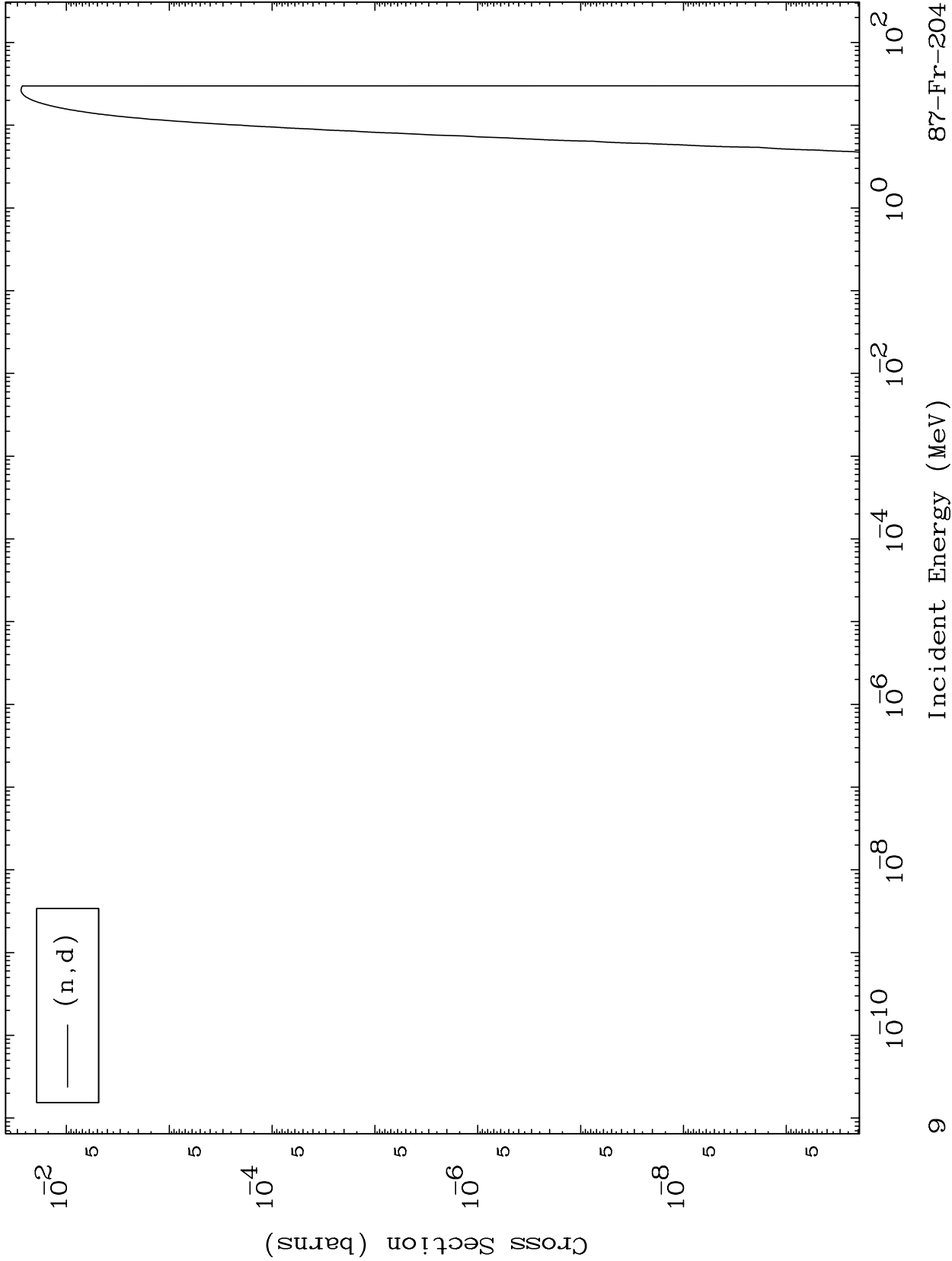
87-Fr-204



MAT 8702

(n,d) Levels  
293 Kelvin Cross Sections

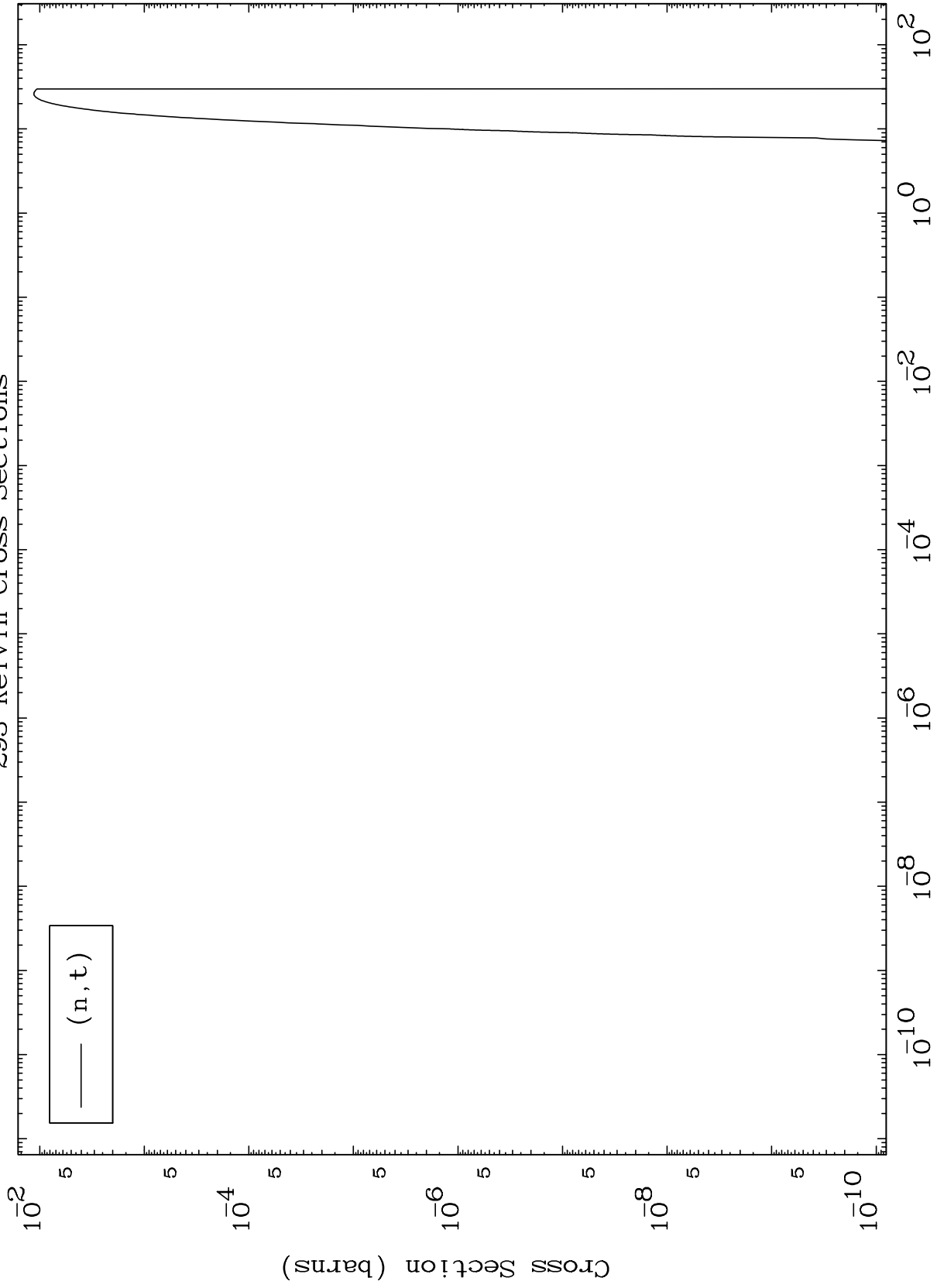
87-Fr-204



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(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-204



10

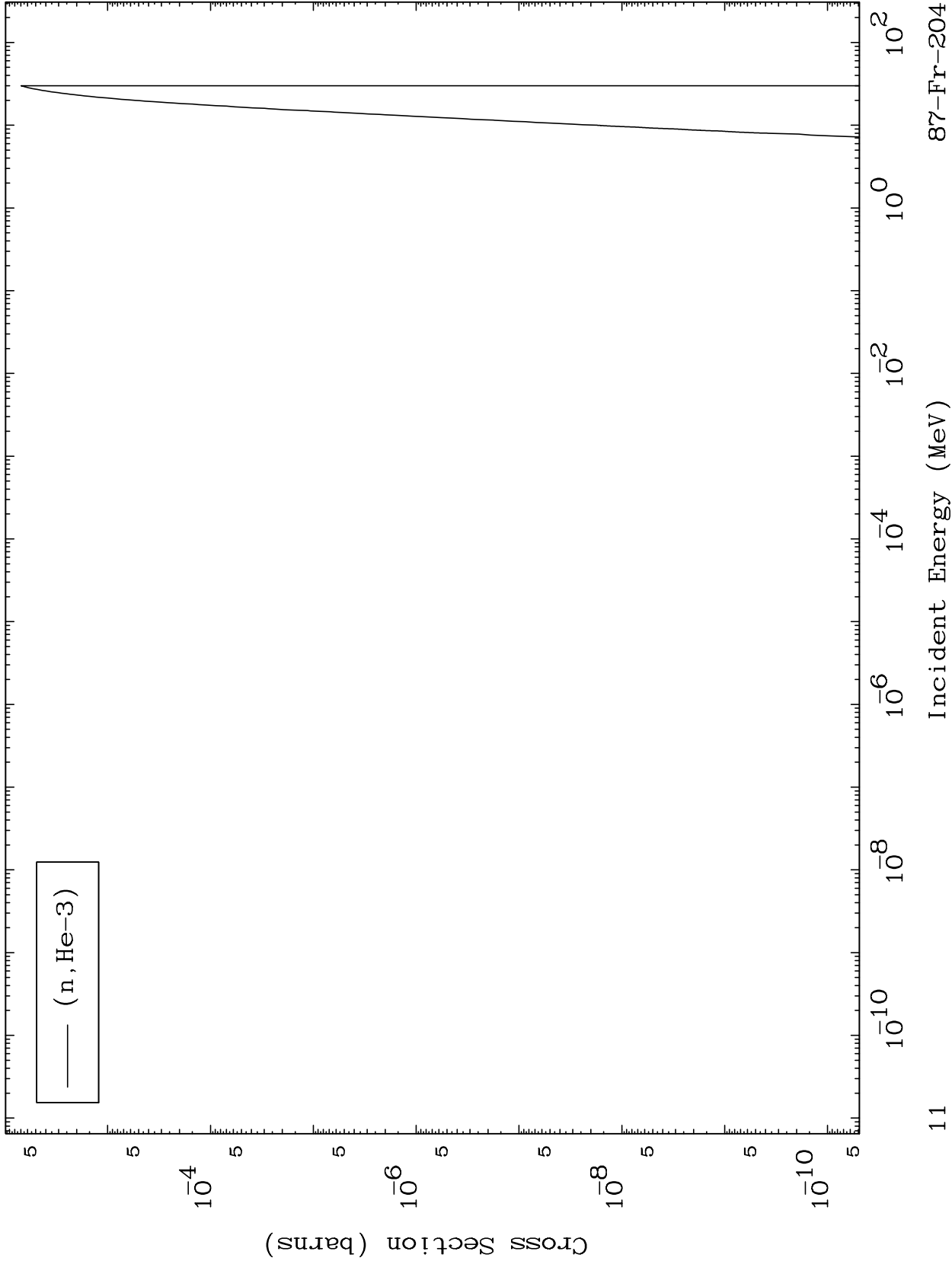
87-Fr-204

87-Fr-204

MAT 8702

(n,He3) Levels  
293 Kelvin Cross Sections

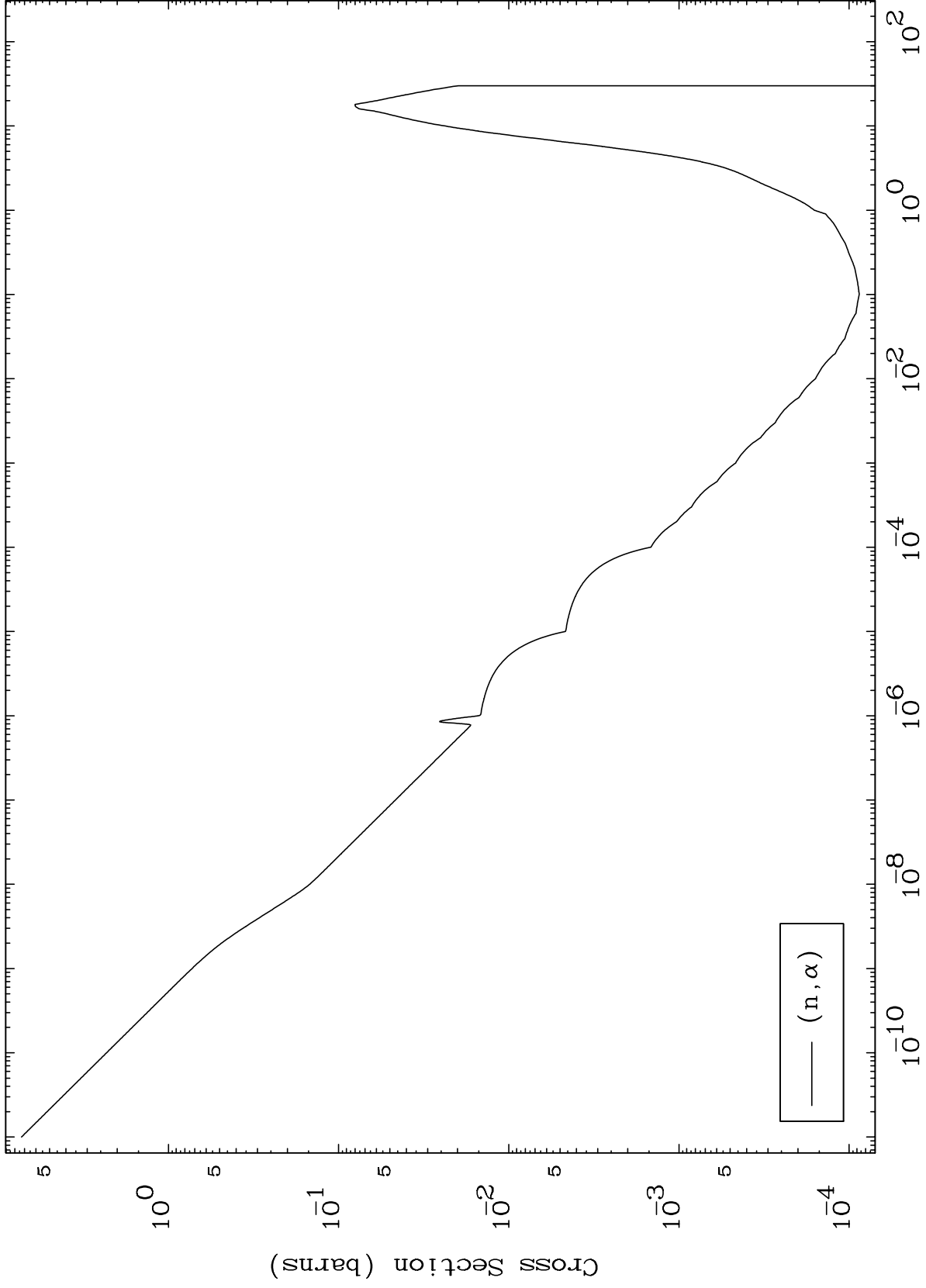
87-Fr-204



MAT 8702

(n,α) Levels  
293 Kelvin Cross Sections

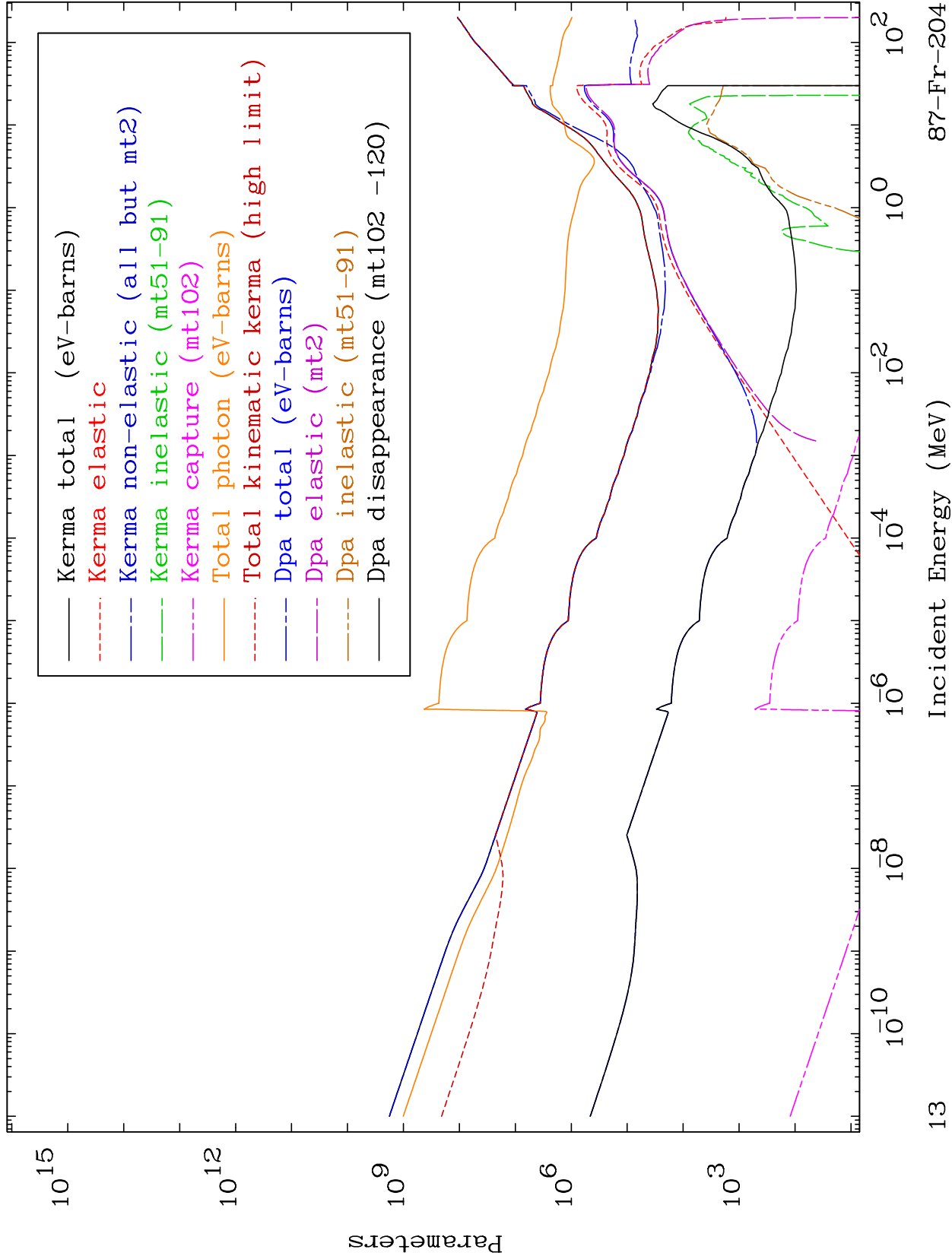
87-Fr-204

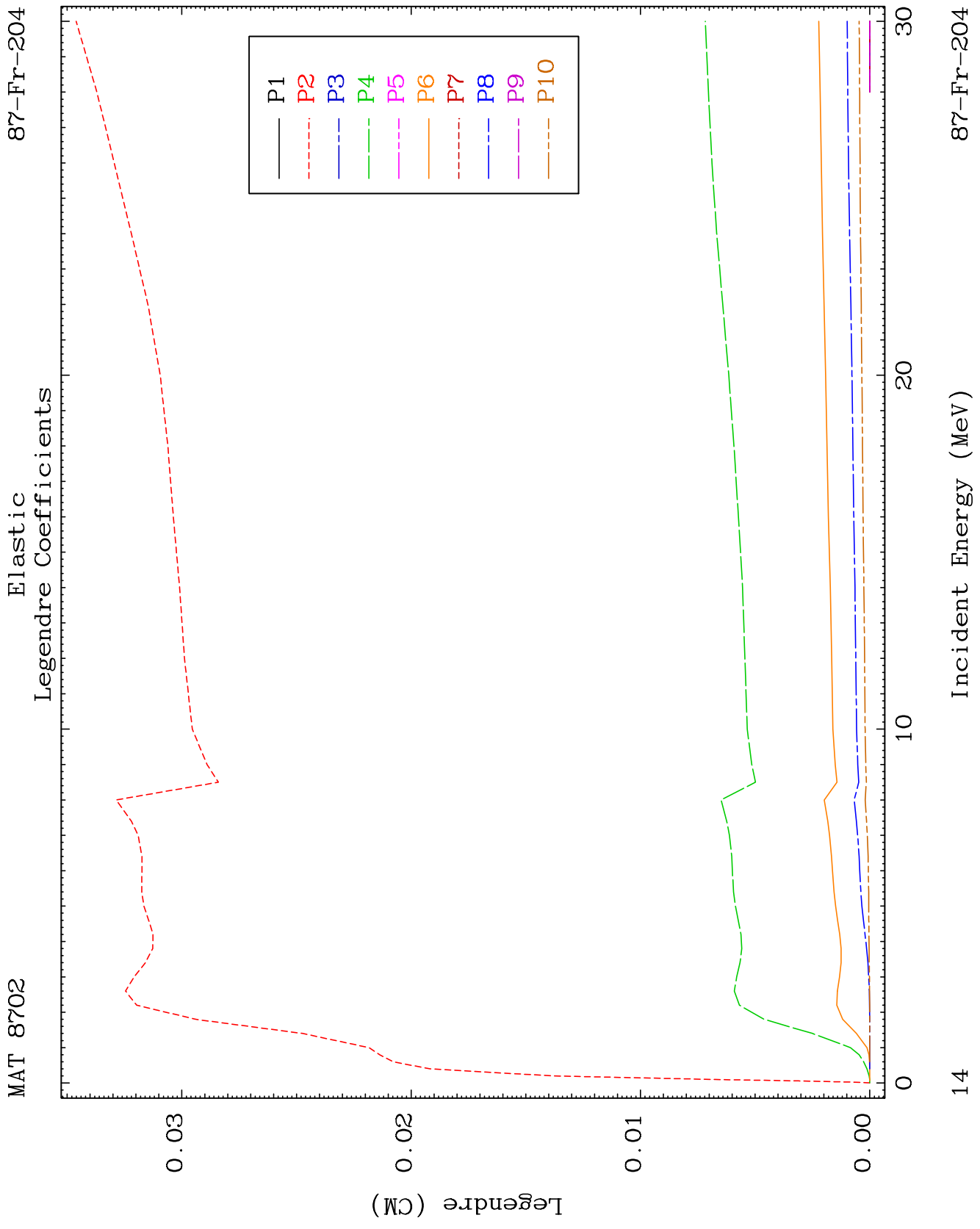


12

Incident Energy (MeV)

87-Fr-204

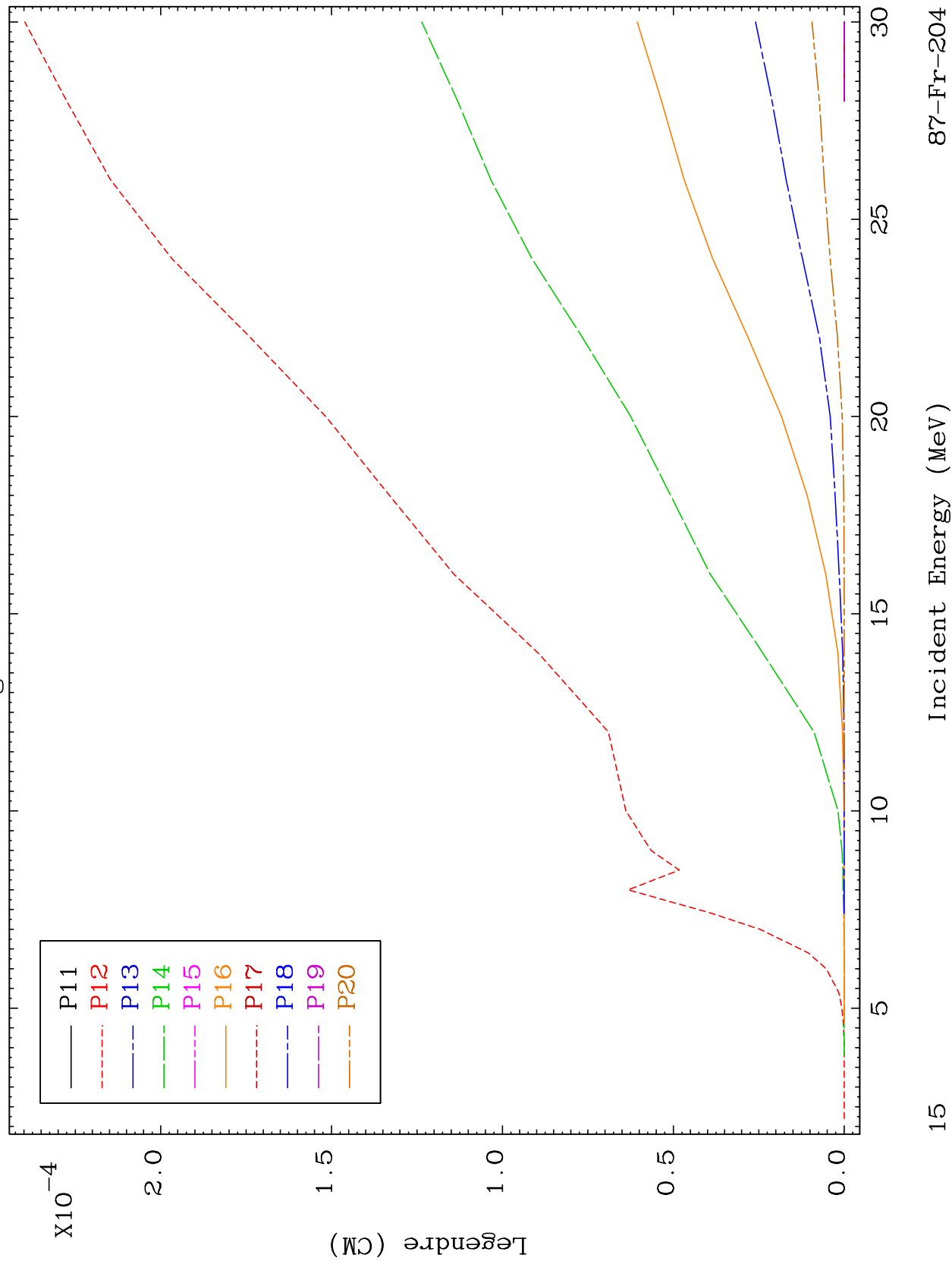




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Elastic Legendre Coefficients

87-Fr-204



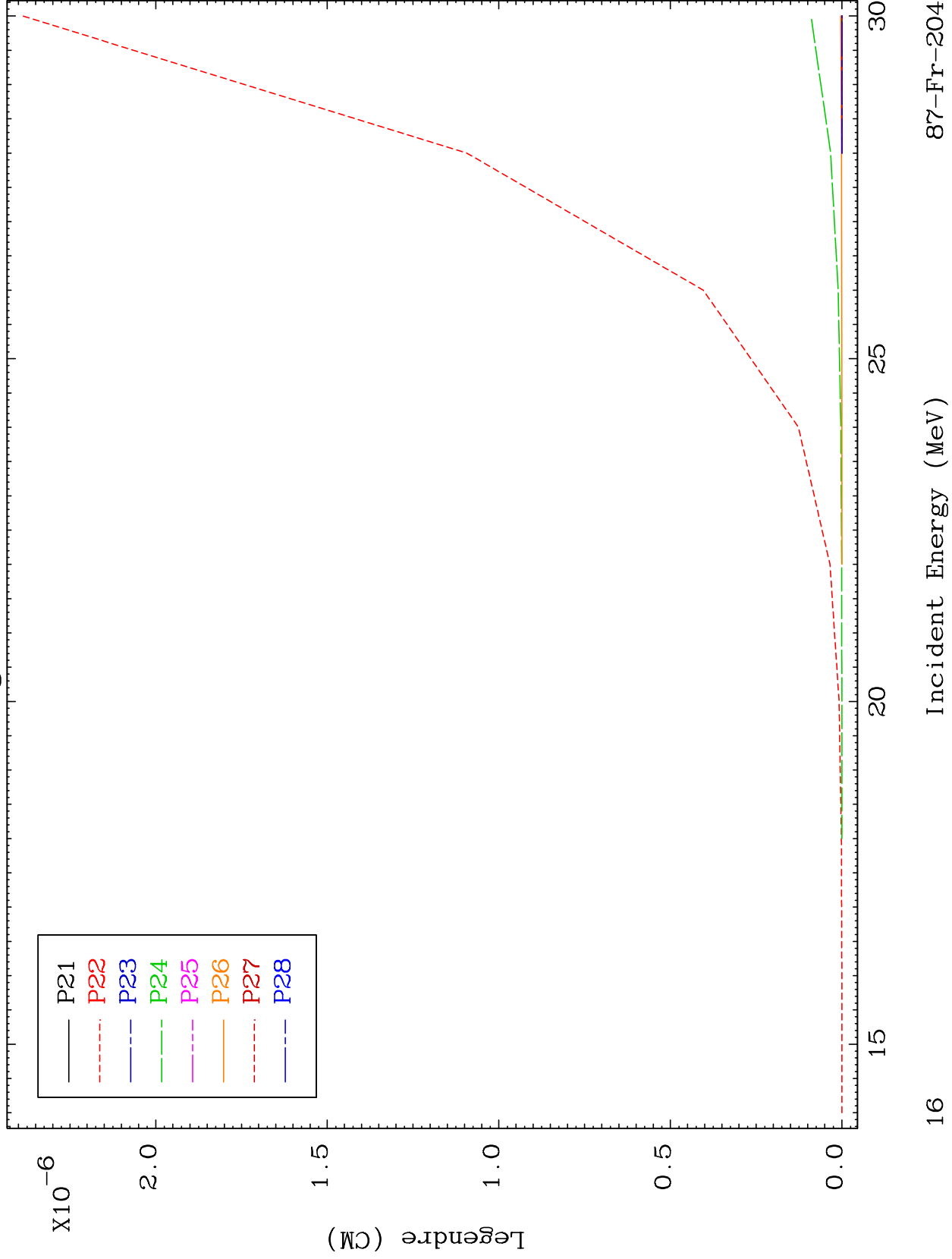
- P11
- P12
- P13
- P14
- P15
- P16
- P17
- P18
- P19
- P20



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Elastic Legendre Coefficients

87-Fr-204



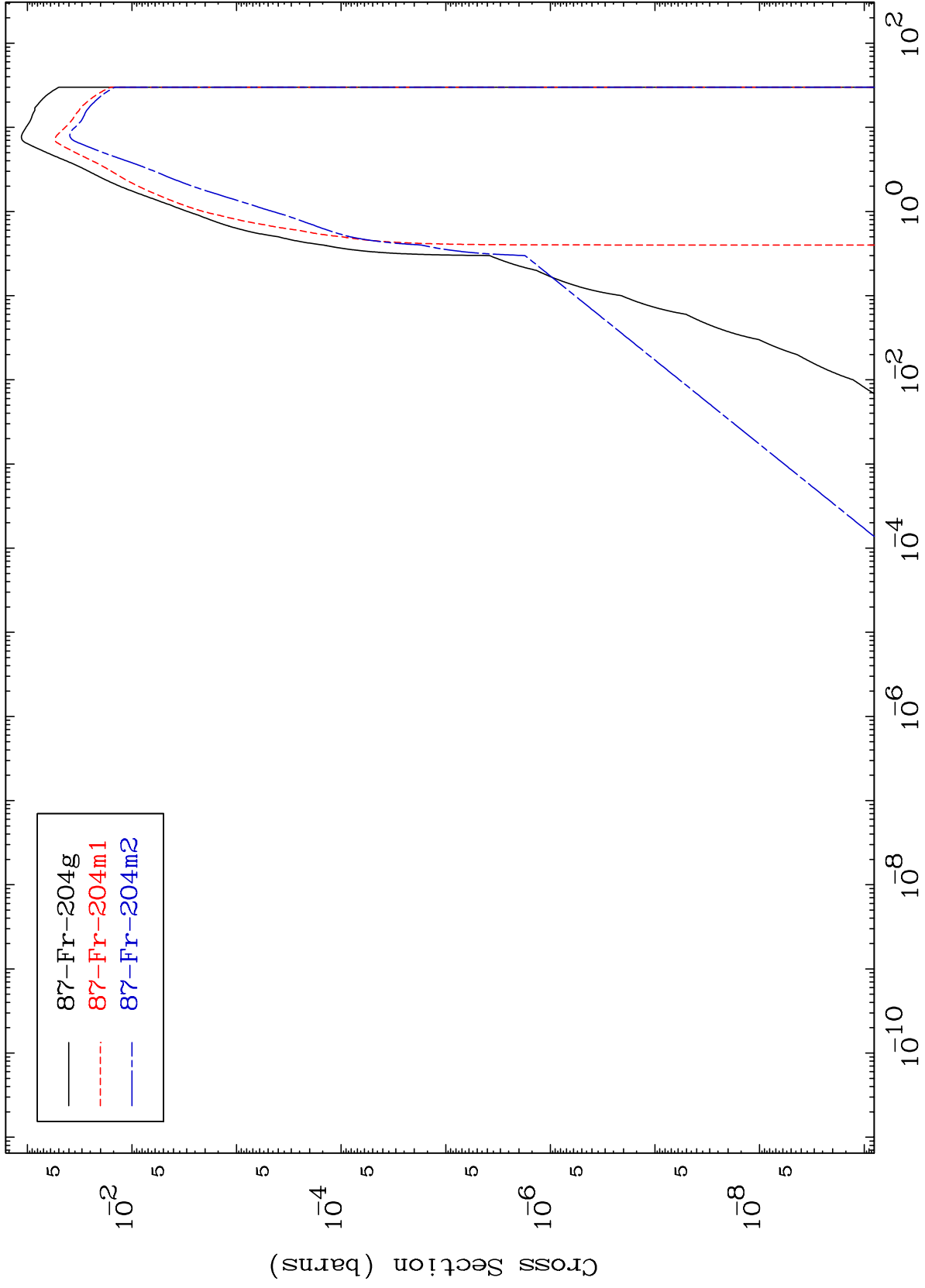
16

87-Fr-204

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Inelastic  
Radionuclide Production Cross Section

87-Fr-204

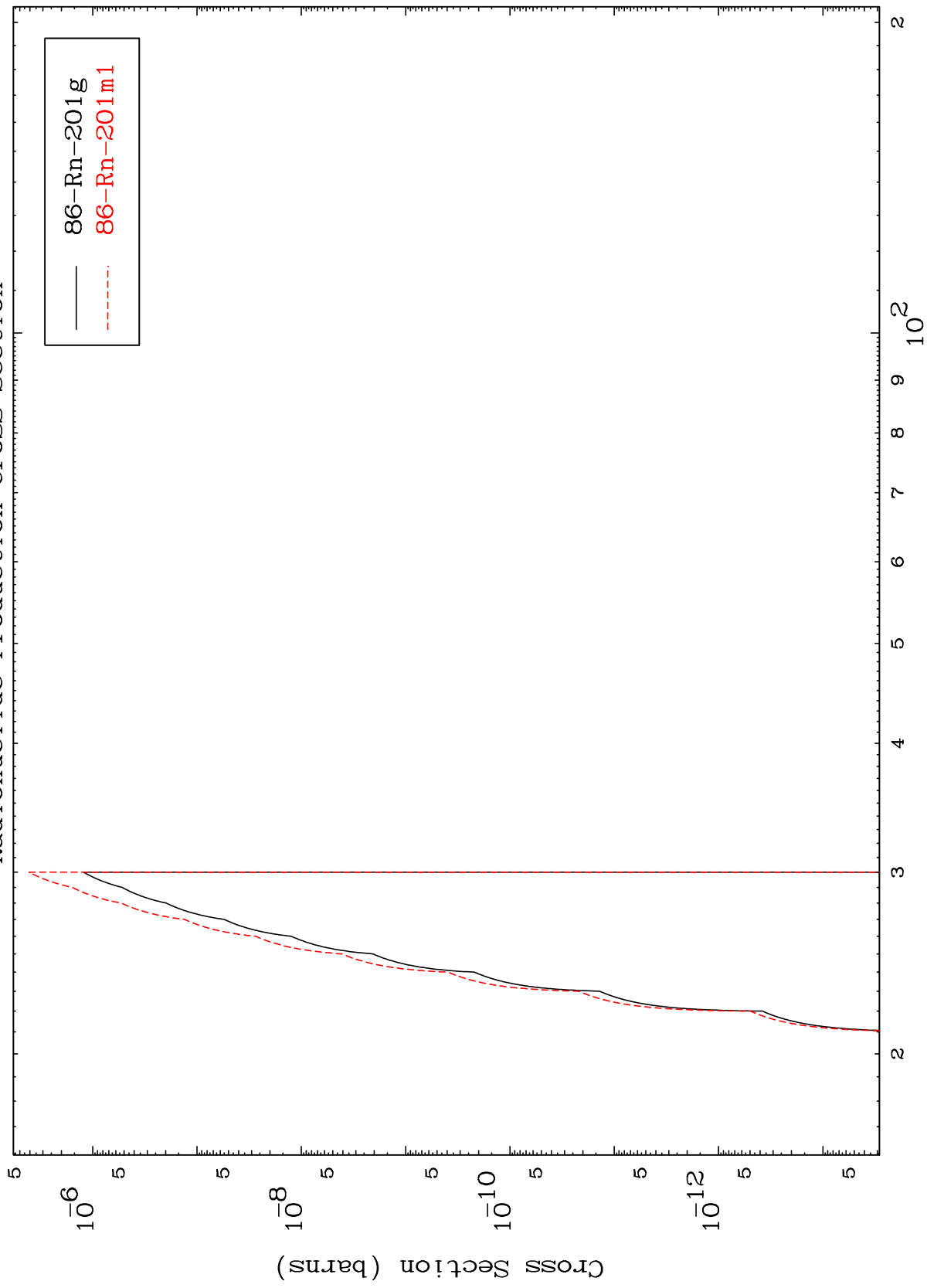


87-Fr-204

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87-Fr-204

(n,2n) d  
Radionuclide Production Cross Section



18

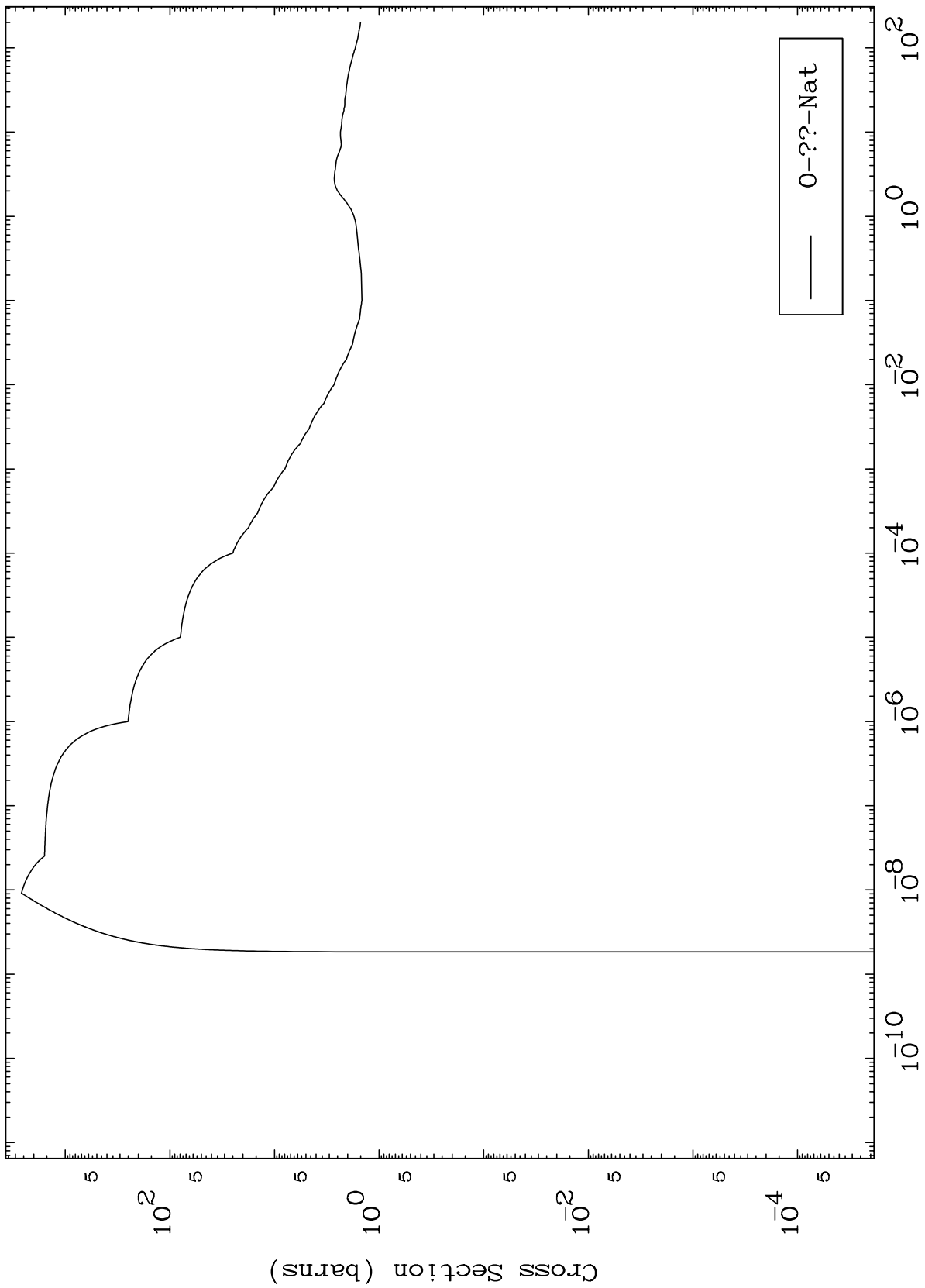
Incident Energy (MeV)

87-Fr-204

MAT 8702

87-Fr-204

Fission  
Radionuclide Production Cross Section

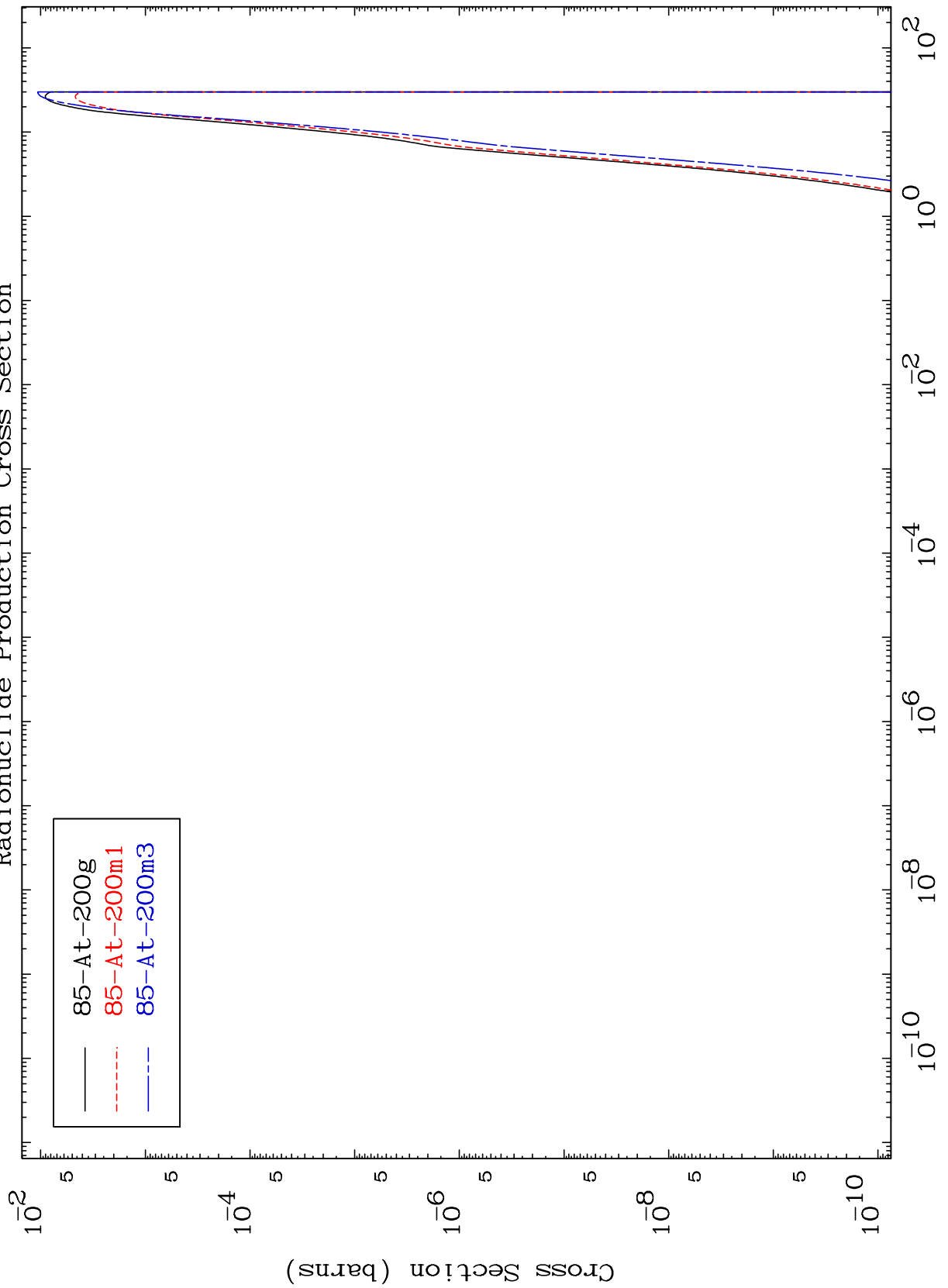


MAT 8702

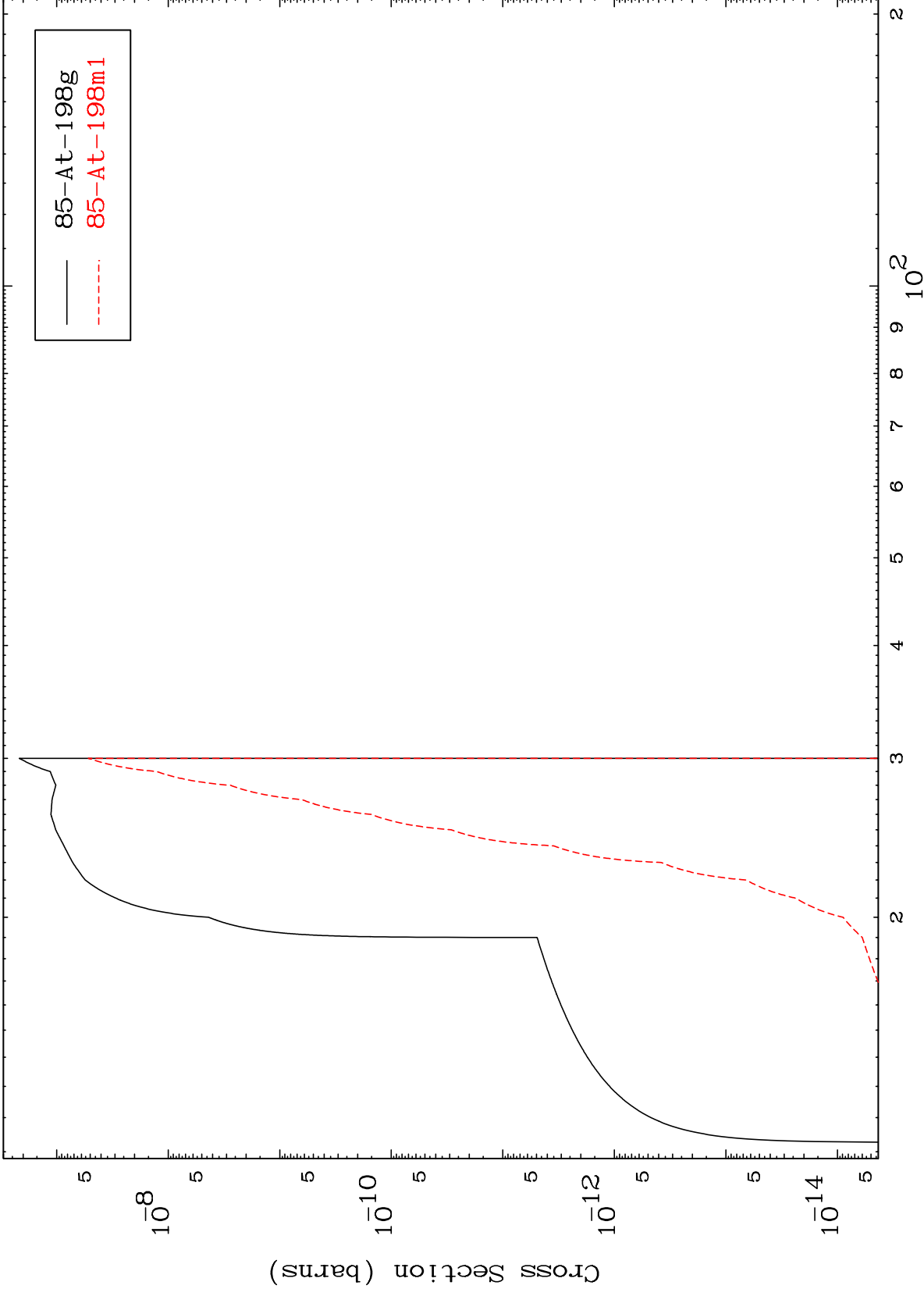
$(n, n') \alpha$

87-Fr-204

Radionuclide Production Cross Section



Radionuclide Production Cross Section

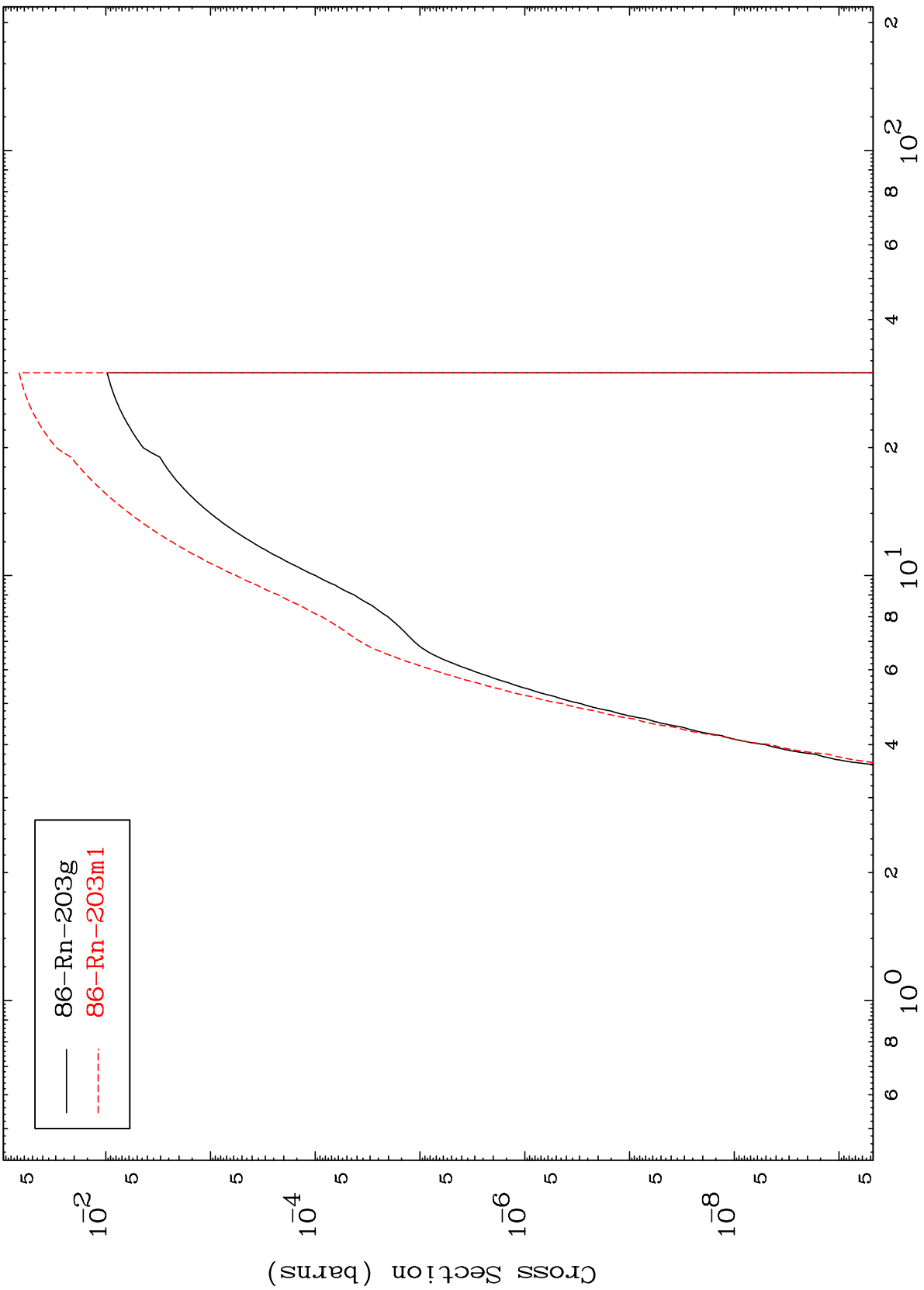


MAT 8702

(n,n') p

87-Fr-204

Radionuclide Production Cross Section



86-Rn-203g  
86-Rn-203m1

22

Incident Energy (MeV)

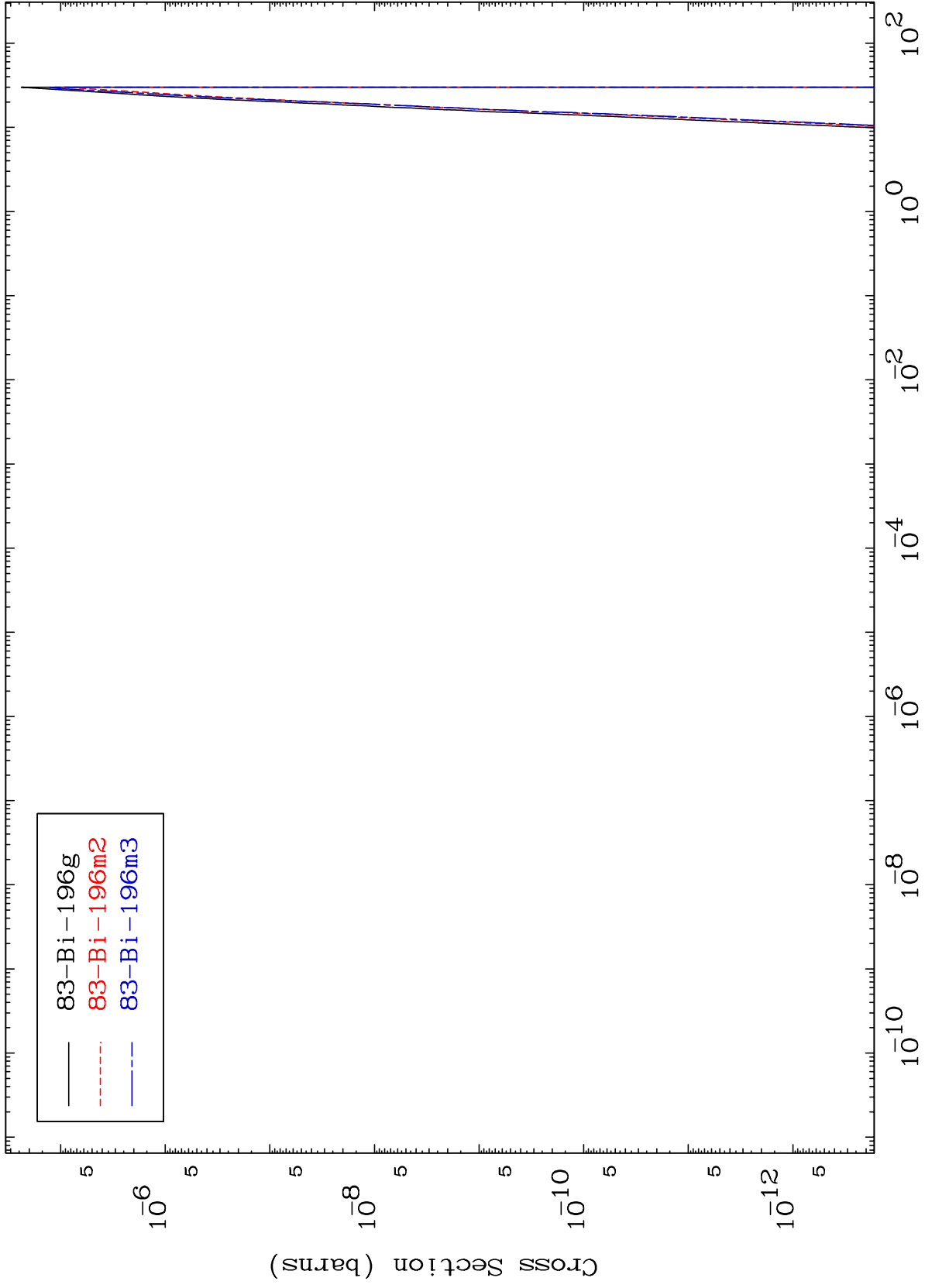
87-Fr-204

MAT 8702

(n,n') 2 $\alpha$

87-Fr-204

Radionuclide Production Cross Section



23

Incident Energy (MeV)

87-Fr-204

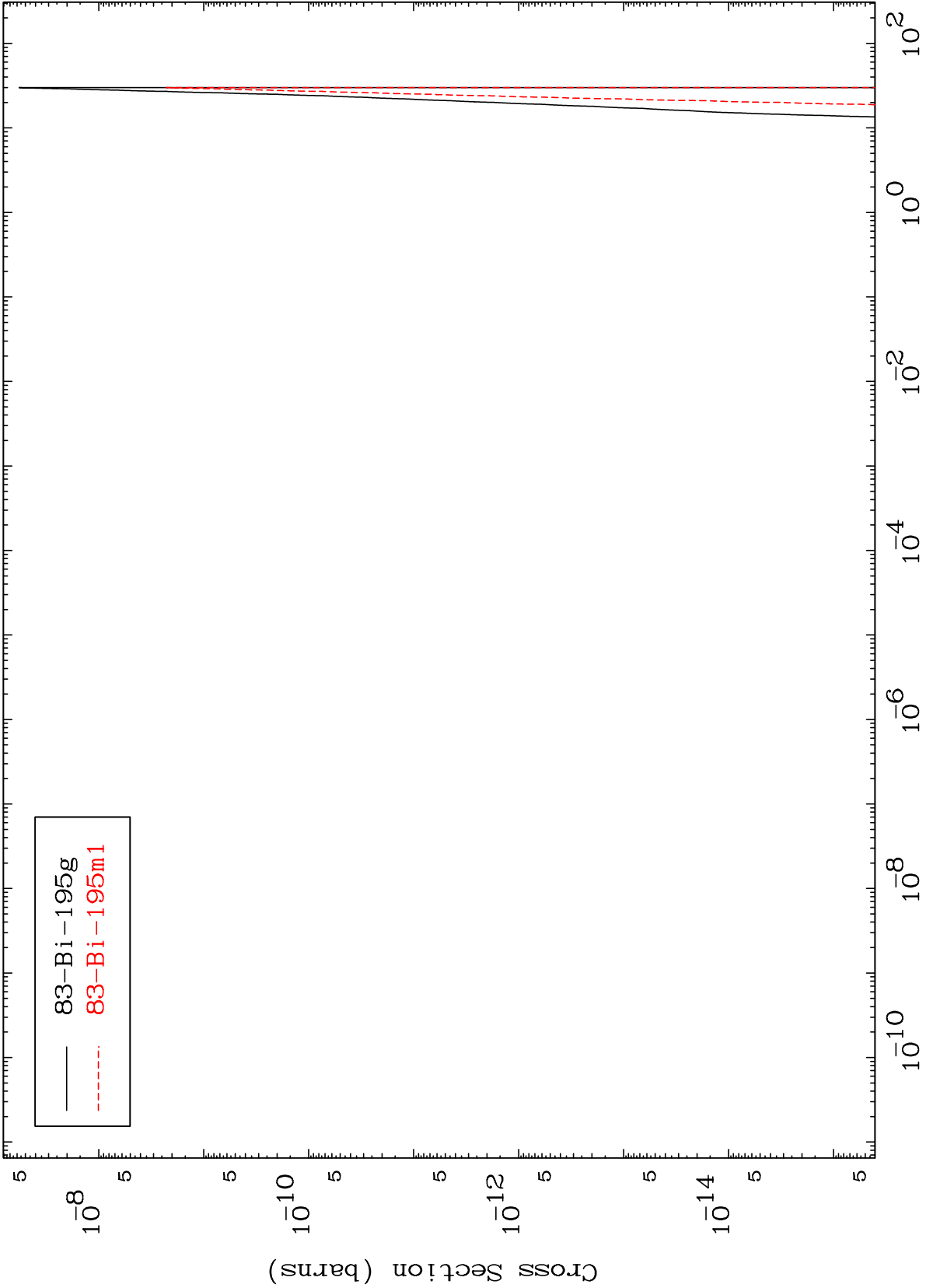


MAT 8702

(n,2n) 2α

87-Fr-204

Radionuclide Production Cross Section



83-Bi-195g  
83-Bi-195m1

Incident Energy (MeV)

87-Fr-204

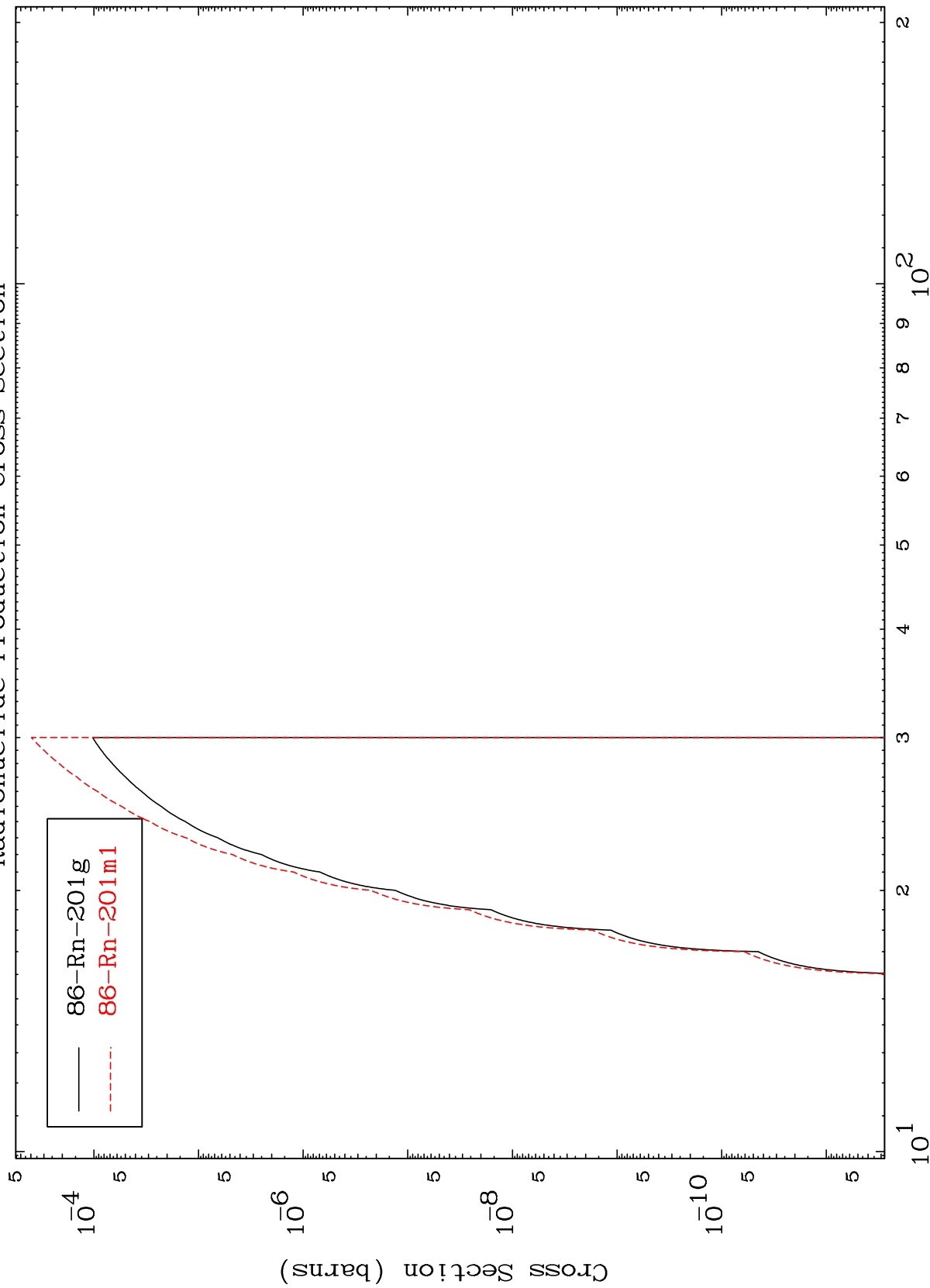
24

MAT 8702

(n,n') t

87-Fr-204

Radionuclide Production Cross Section



Incident Energy (MeV)

87-Fr-204

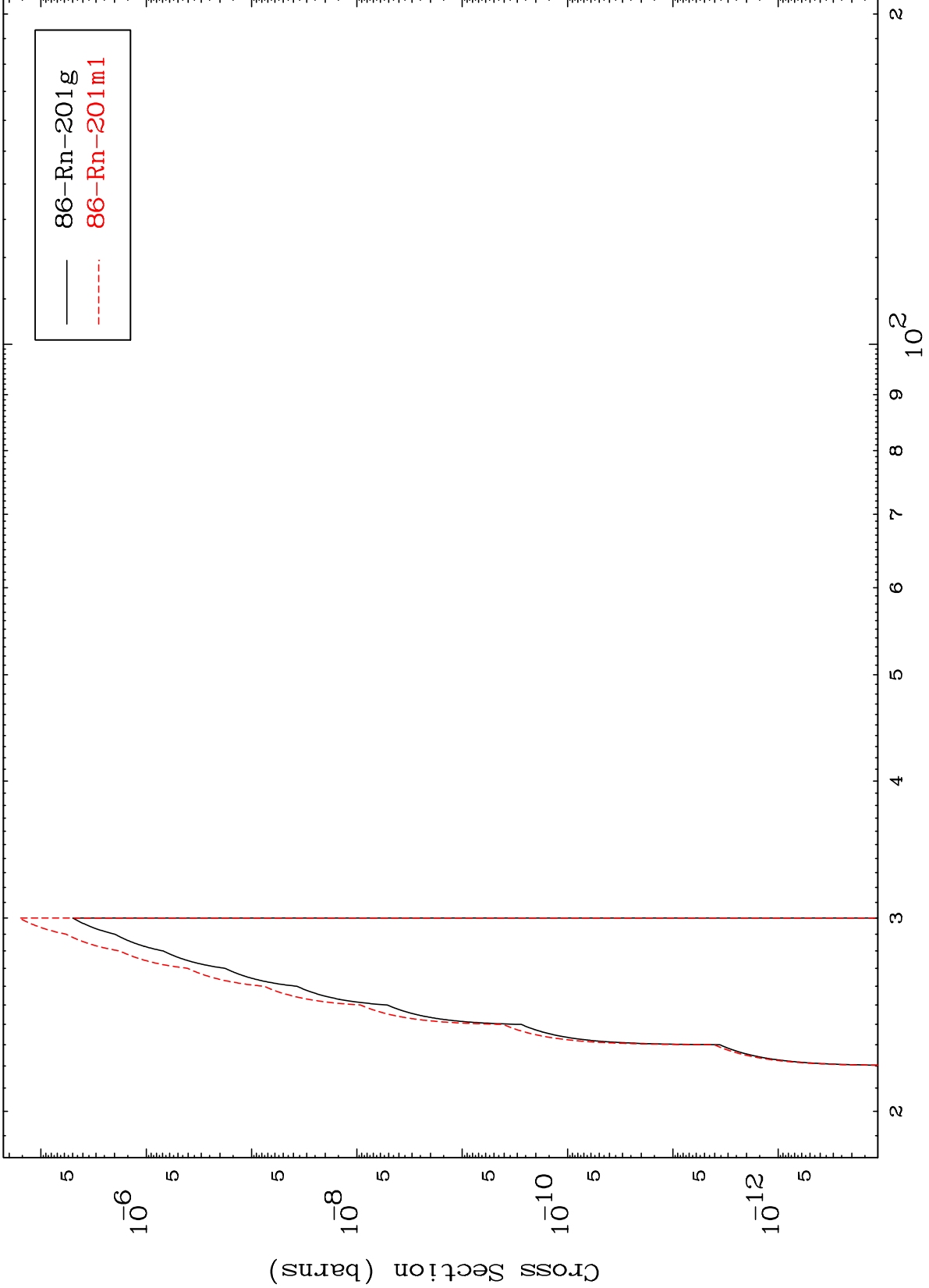
25

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(n,3n) p

87-Fr-204

Radionuclide Production Cross Section



26

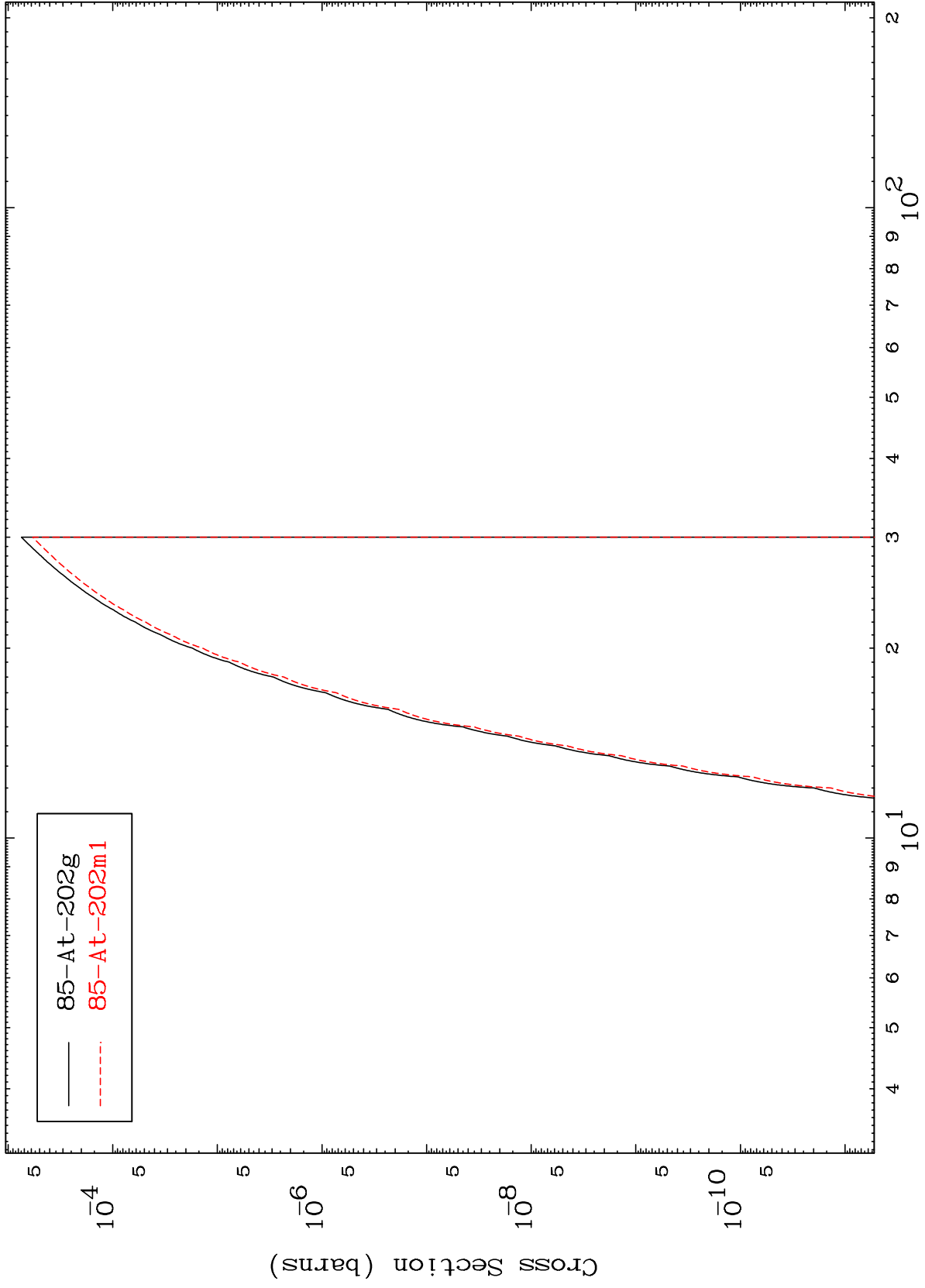
Incident Energy (MeV)

87-Fr-204

MAT 8702

87-Fr-204

(n,2n) p  
Radionuclide Production Cross Section



27

Incident Energy (MeV)

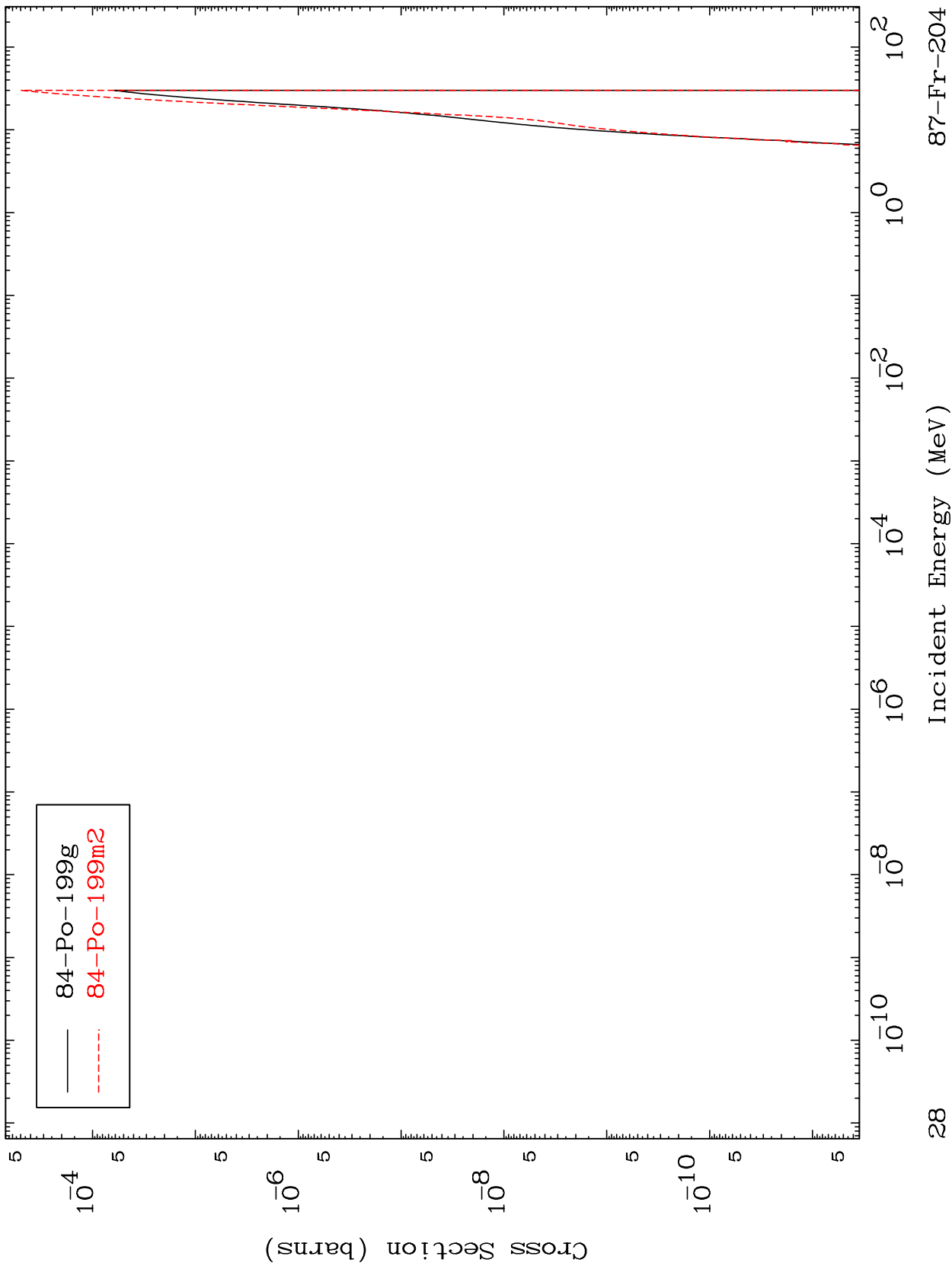
87-Fr-204

MAT 8702

(n,n') p  $\alpha$

87-Fr-204

Radionuclide Production Cross Section



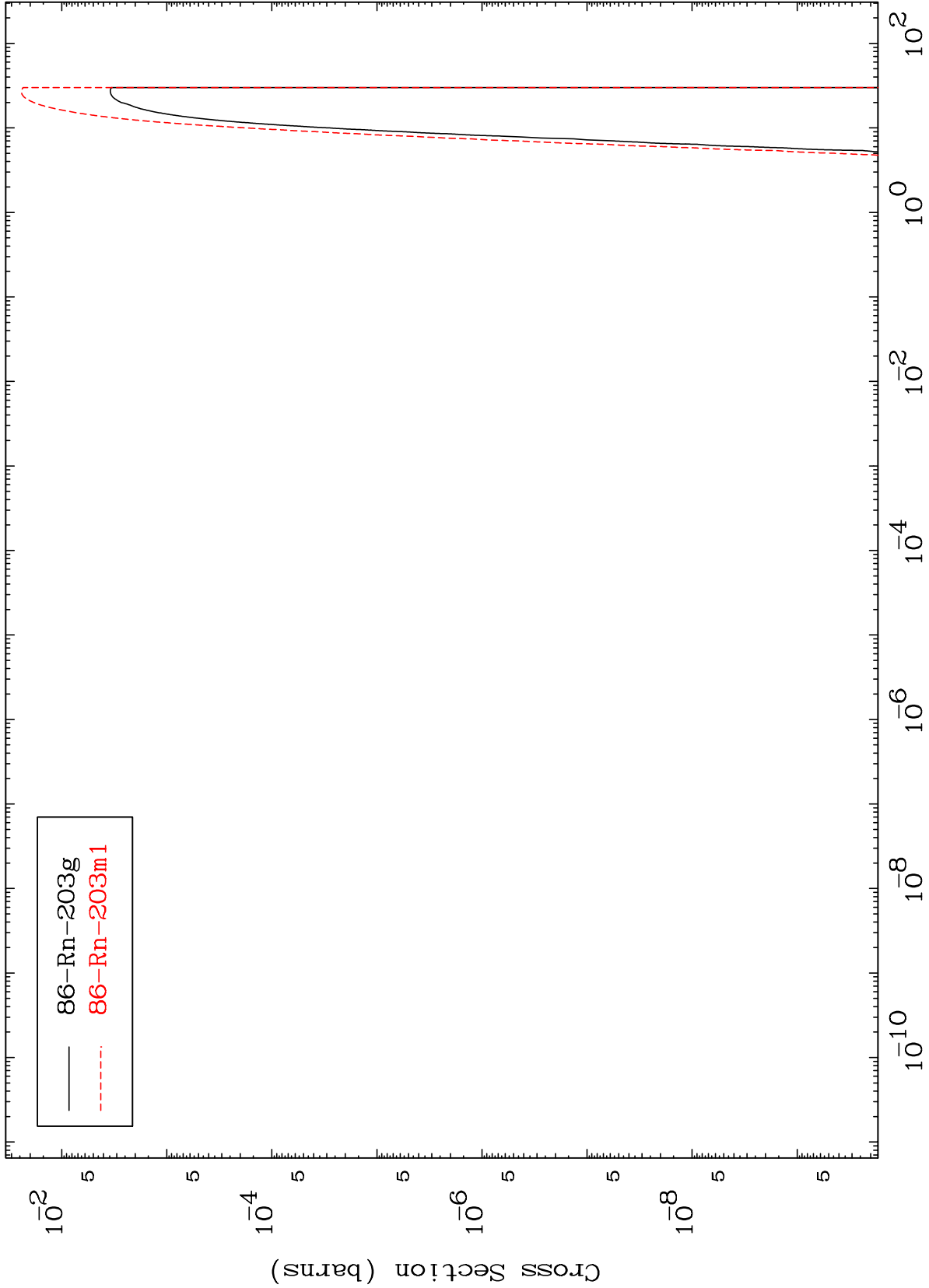
84-Po-199g  
84-Po-199m2

MAT 8702

(n,d)

87-Fr-204

Radionuclide Production Cross Section



— 86-Rn-203g  
- - - 86-Rn-203m1

Incident Energy (MeV)

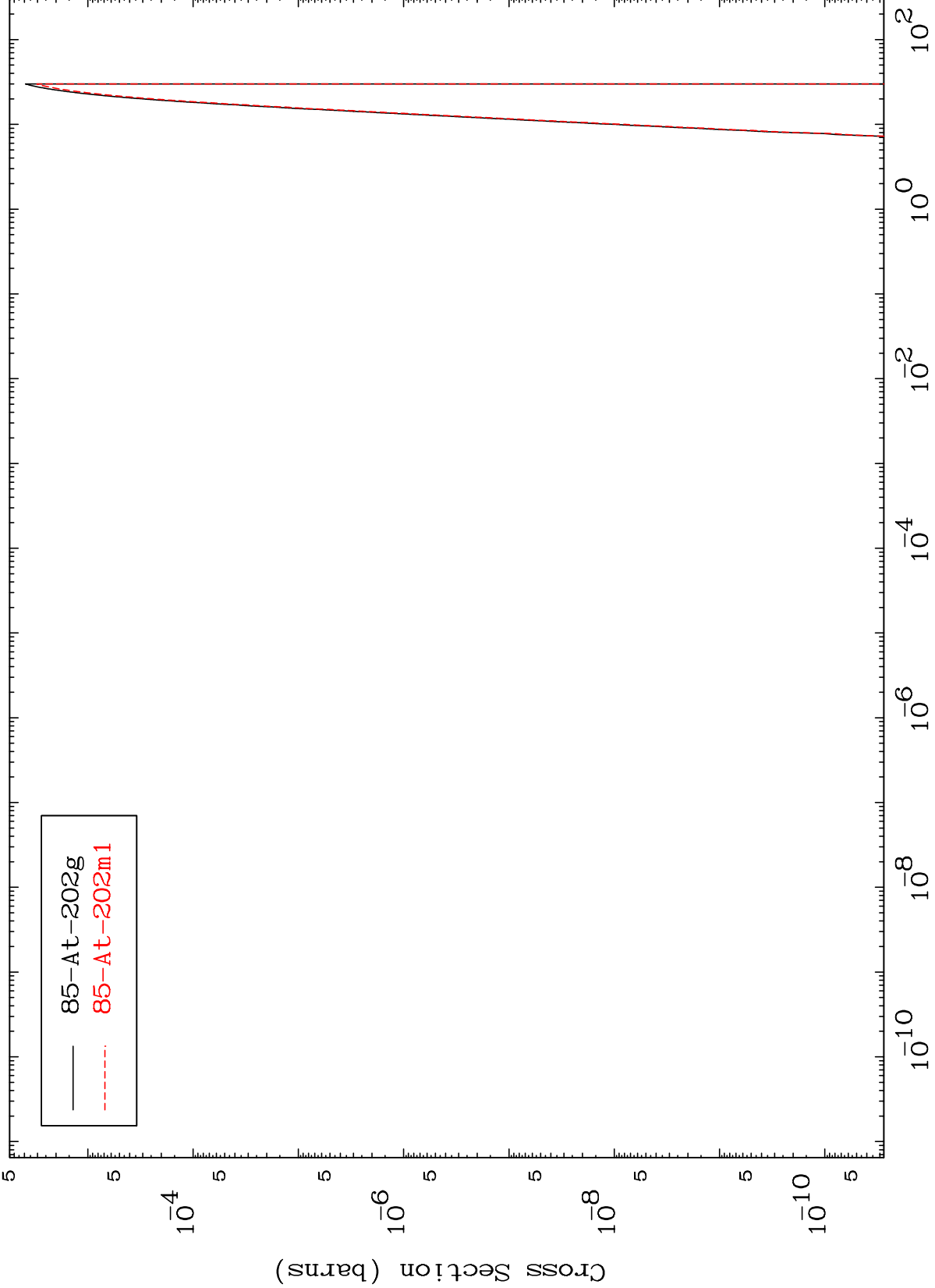
87-Fr-204

MAT 8702

(n,He-3)

87-Fr-204

Radionuclide Production Cross Section



30

Incident Energy (MeV)

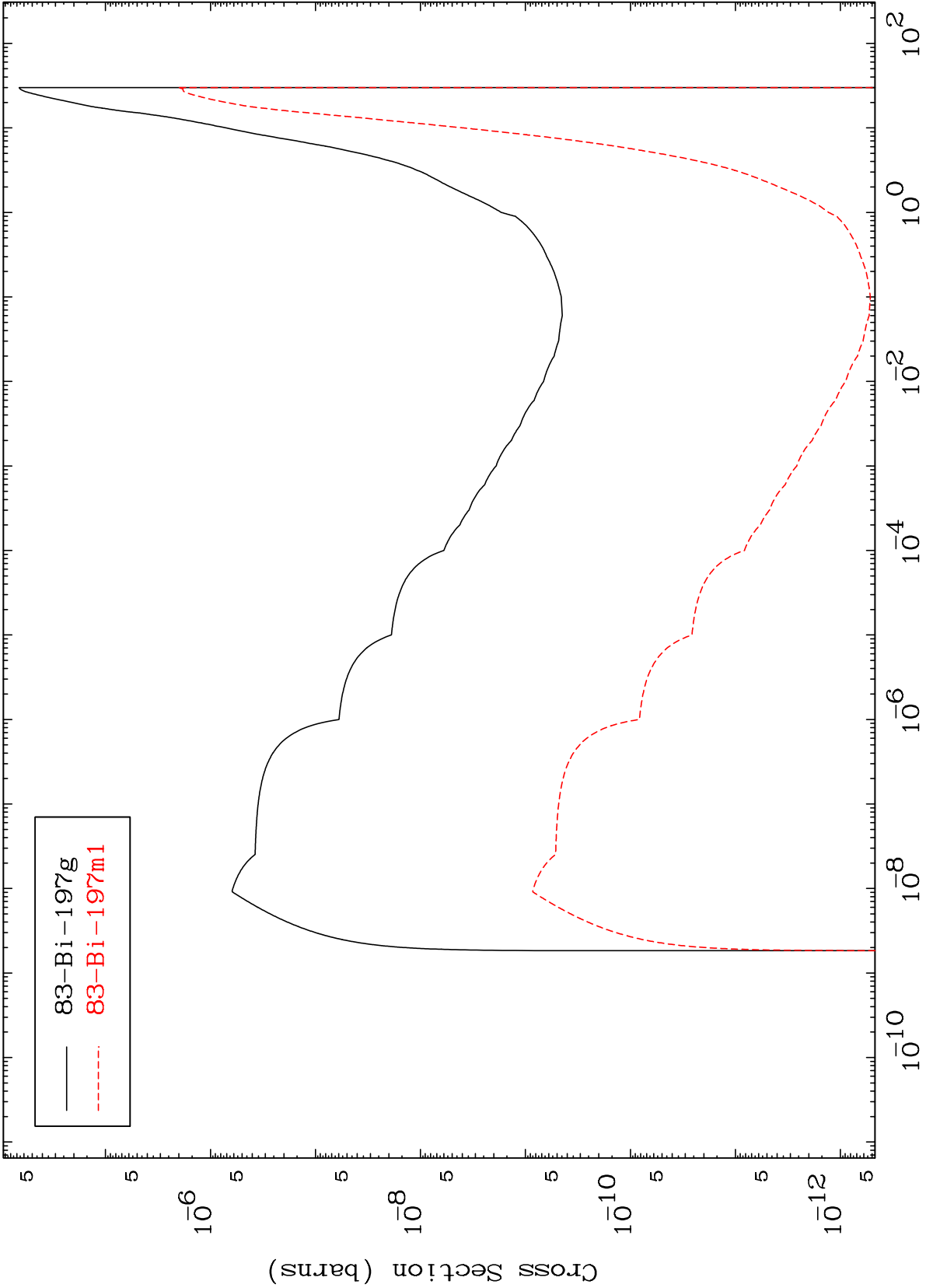
87-Fr-204

MAT 8702

(n,2α)

87-Fr-204

Radionuclide Production Cross Section



83-Bi-197g  
83-Bi-197m1

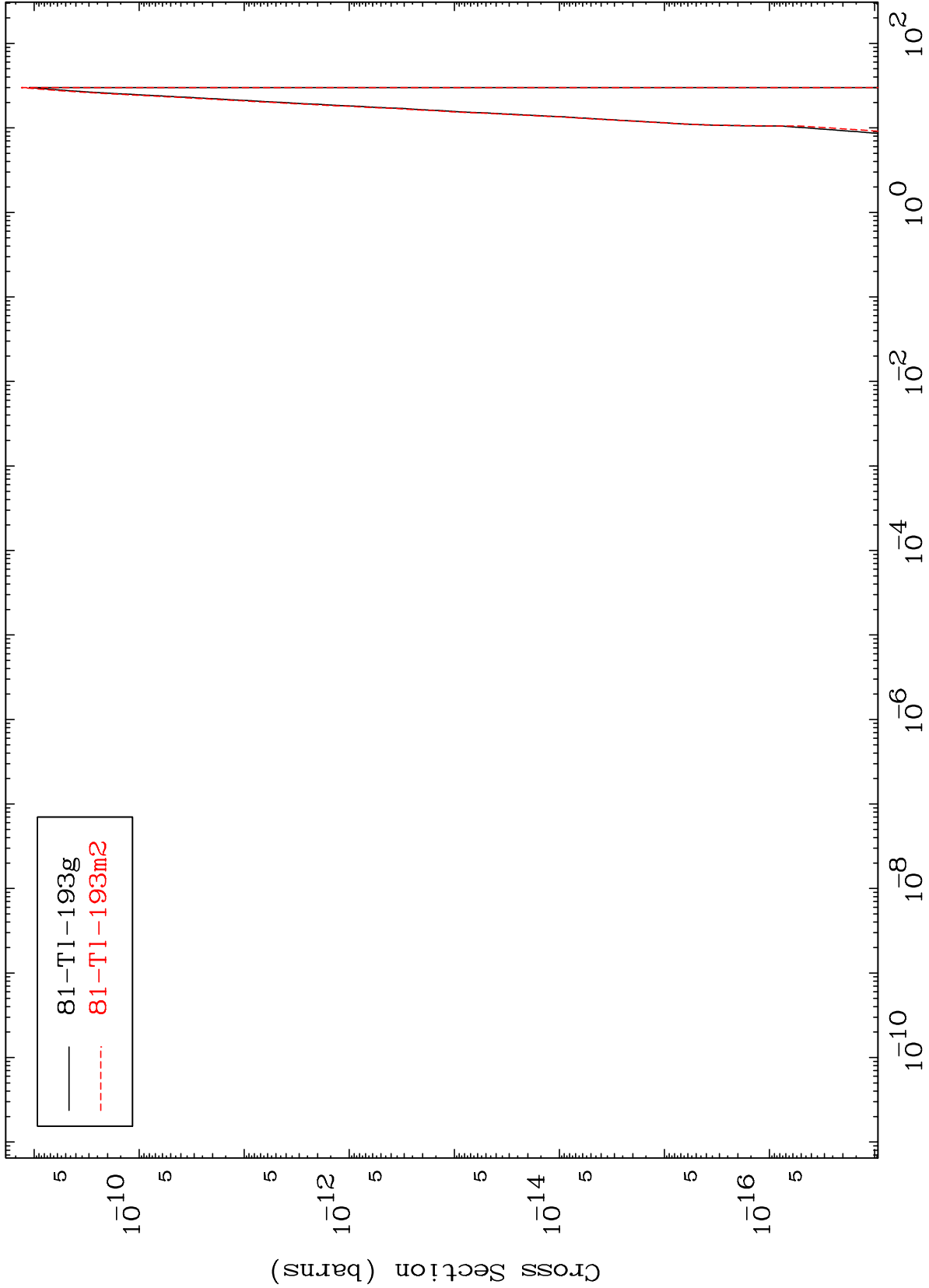


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(n,3 $\alpha$ )

87-Fr-204

Radionuclide Production Cross Section

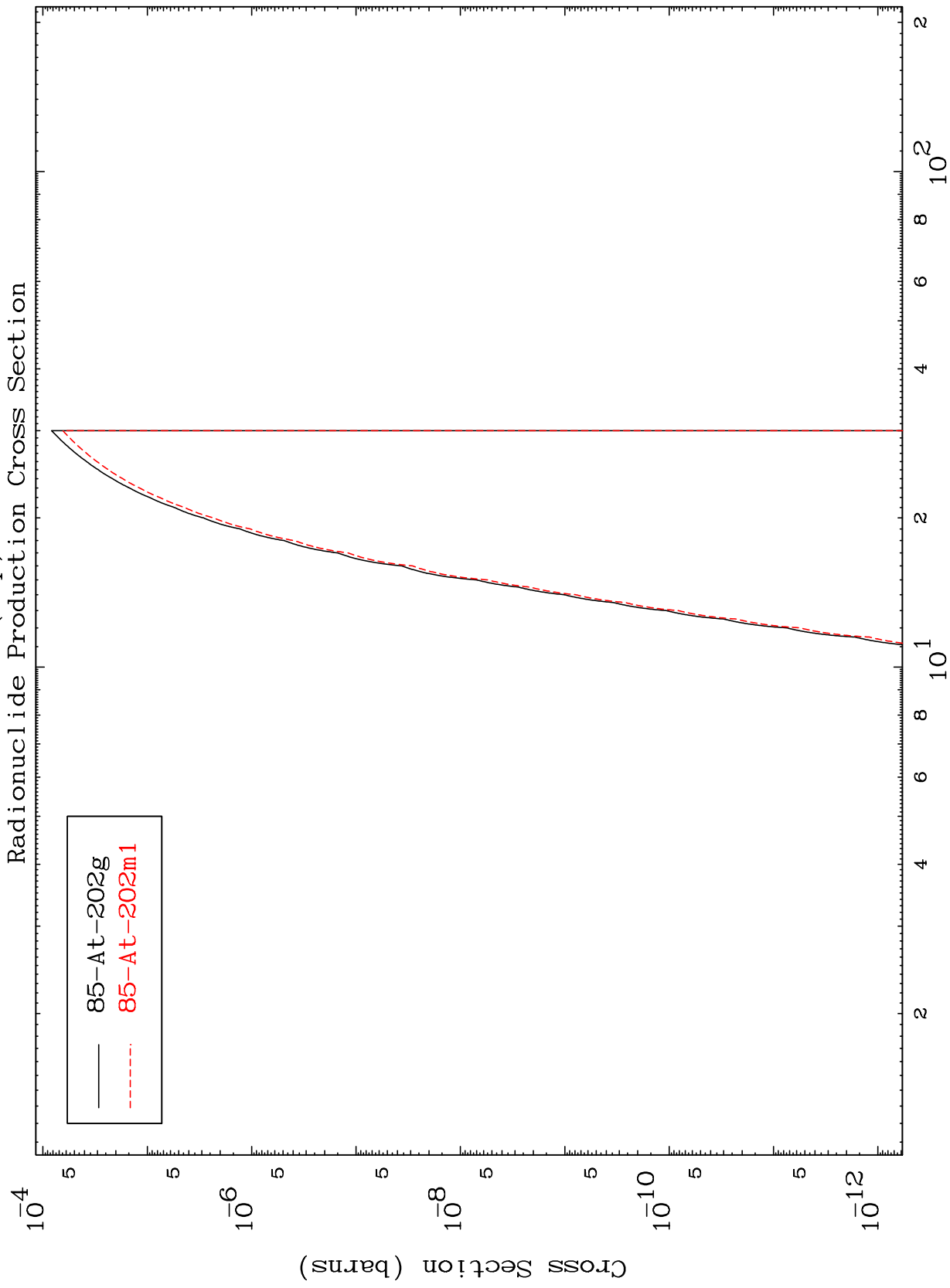


MAT 8702

(n,p) d

87-Fr-204

Radionuclide Production Cross Section



MAT 8702

(n,d)  $\alpha$

87-Fr-204

Radionuclide Production Cross Section

