

Program EVALPLOT
(Version 2018-1)

by

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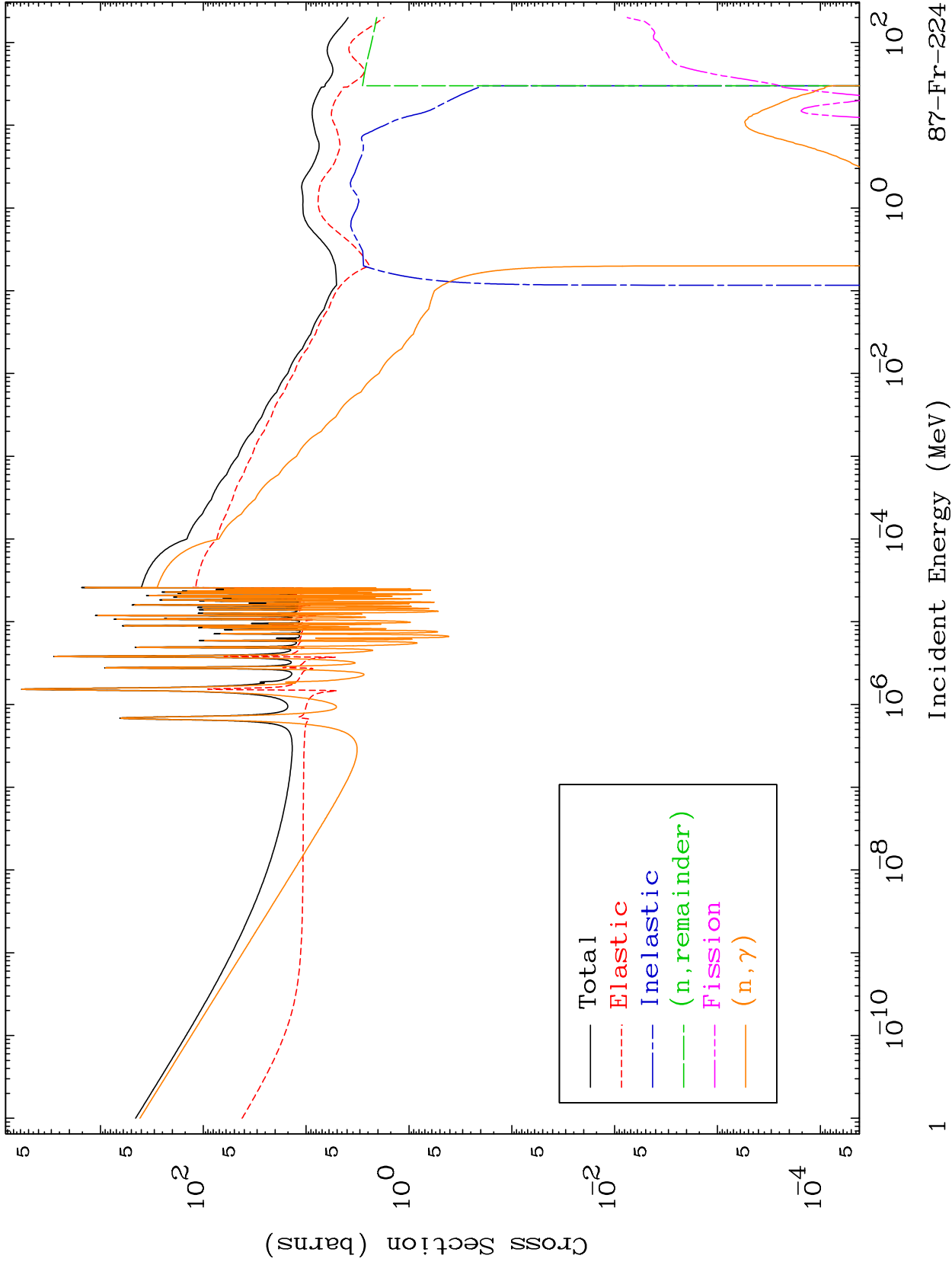
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Press Mouse Button to Start

MAT 8761

Major
293 Kelvin Cross Sections

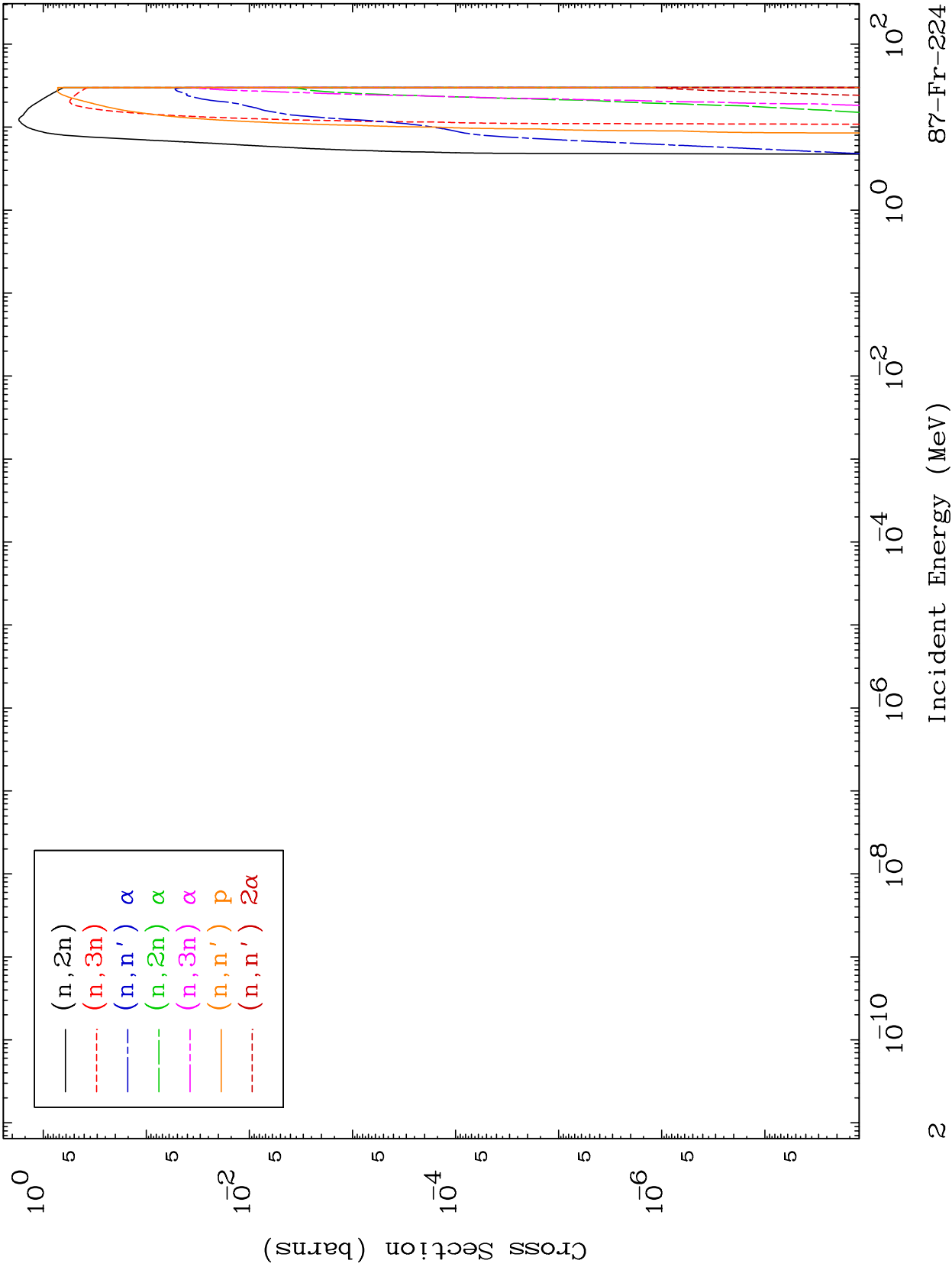
87-Fr-224



MAT 8761

Neutron Production
293 Kelvin Cross Sections

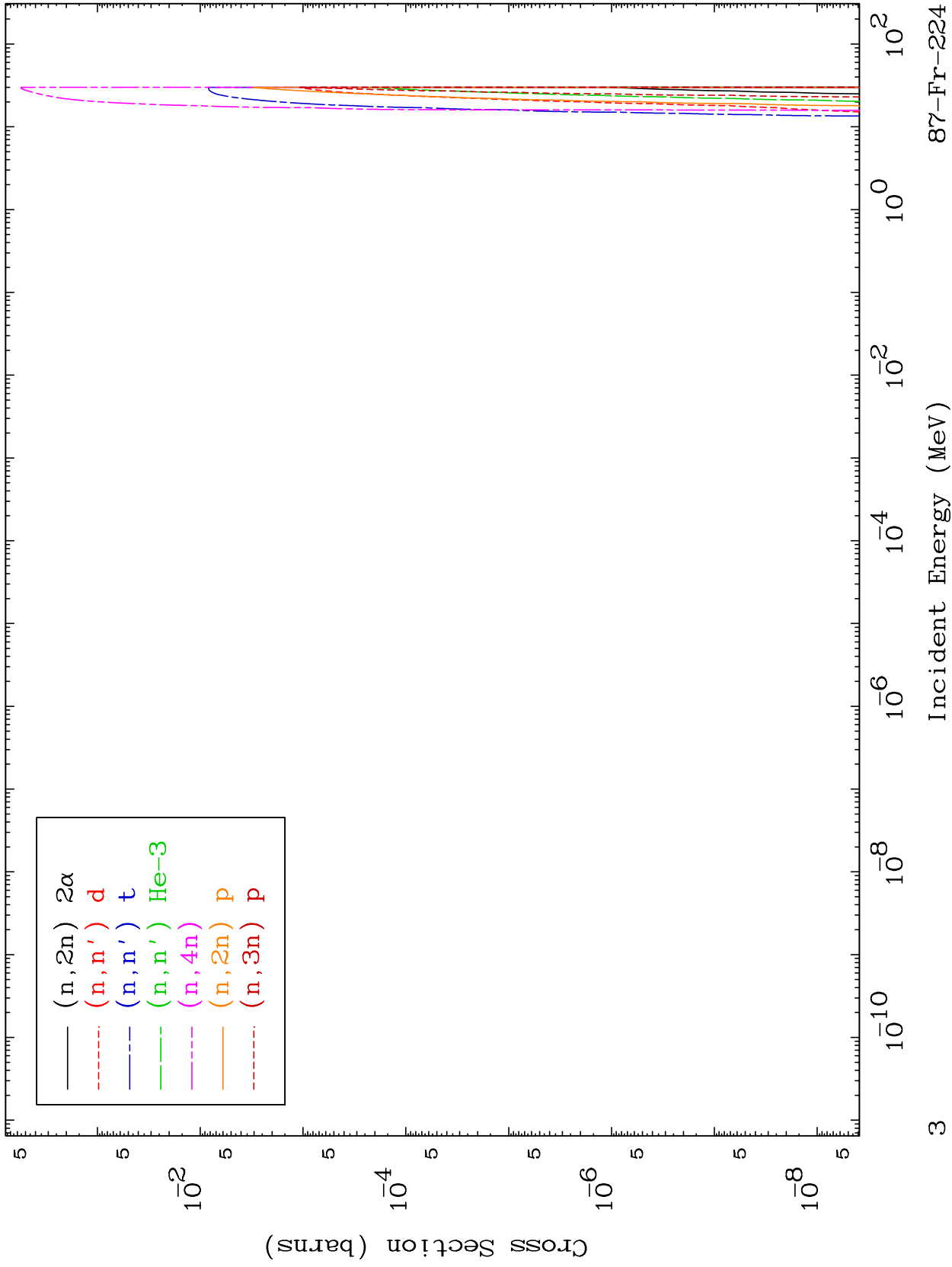
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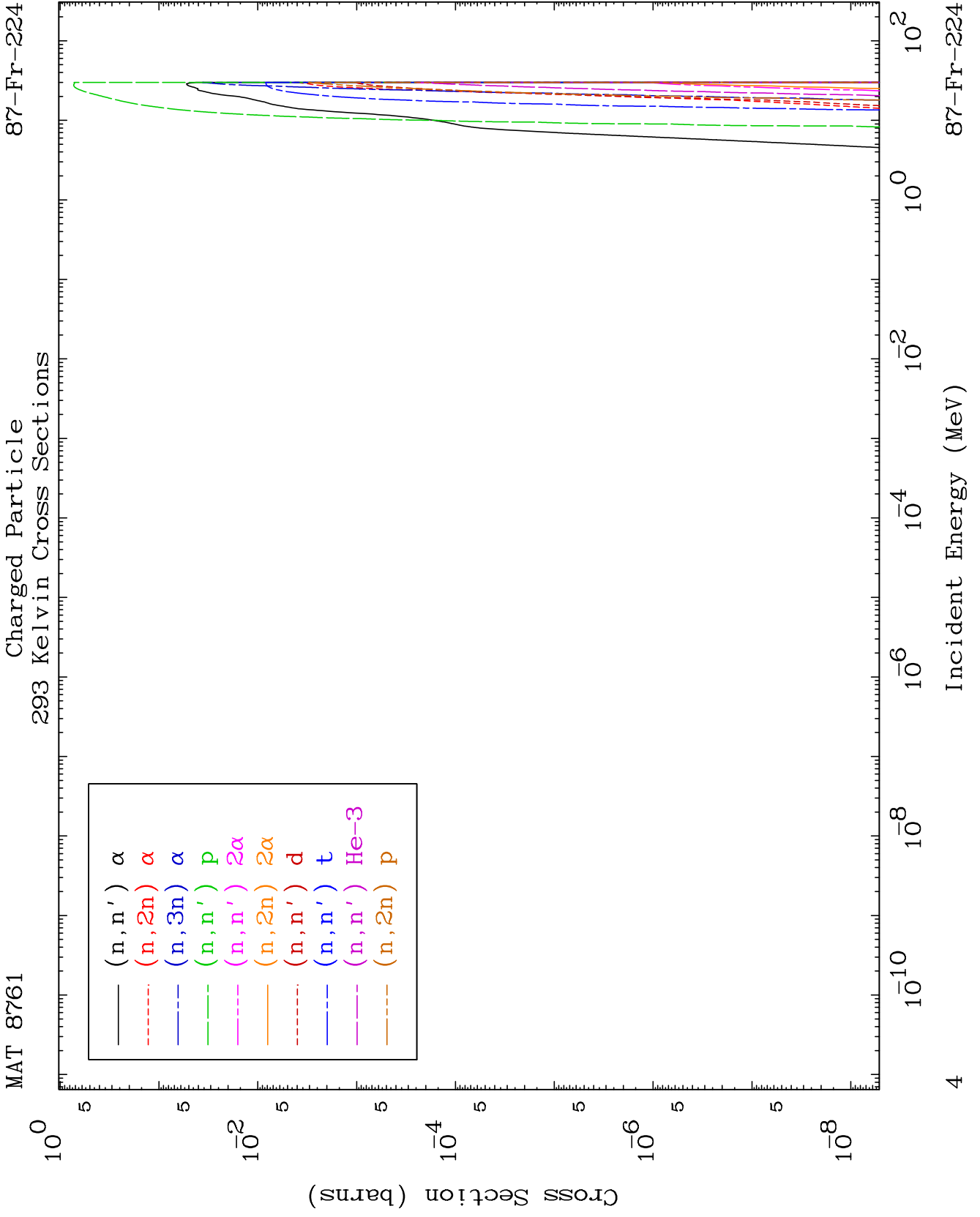


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Neutron Production
293 Kelvin Cross Sections

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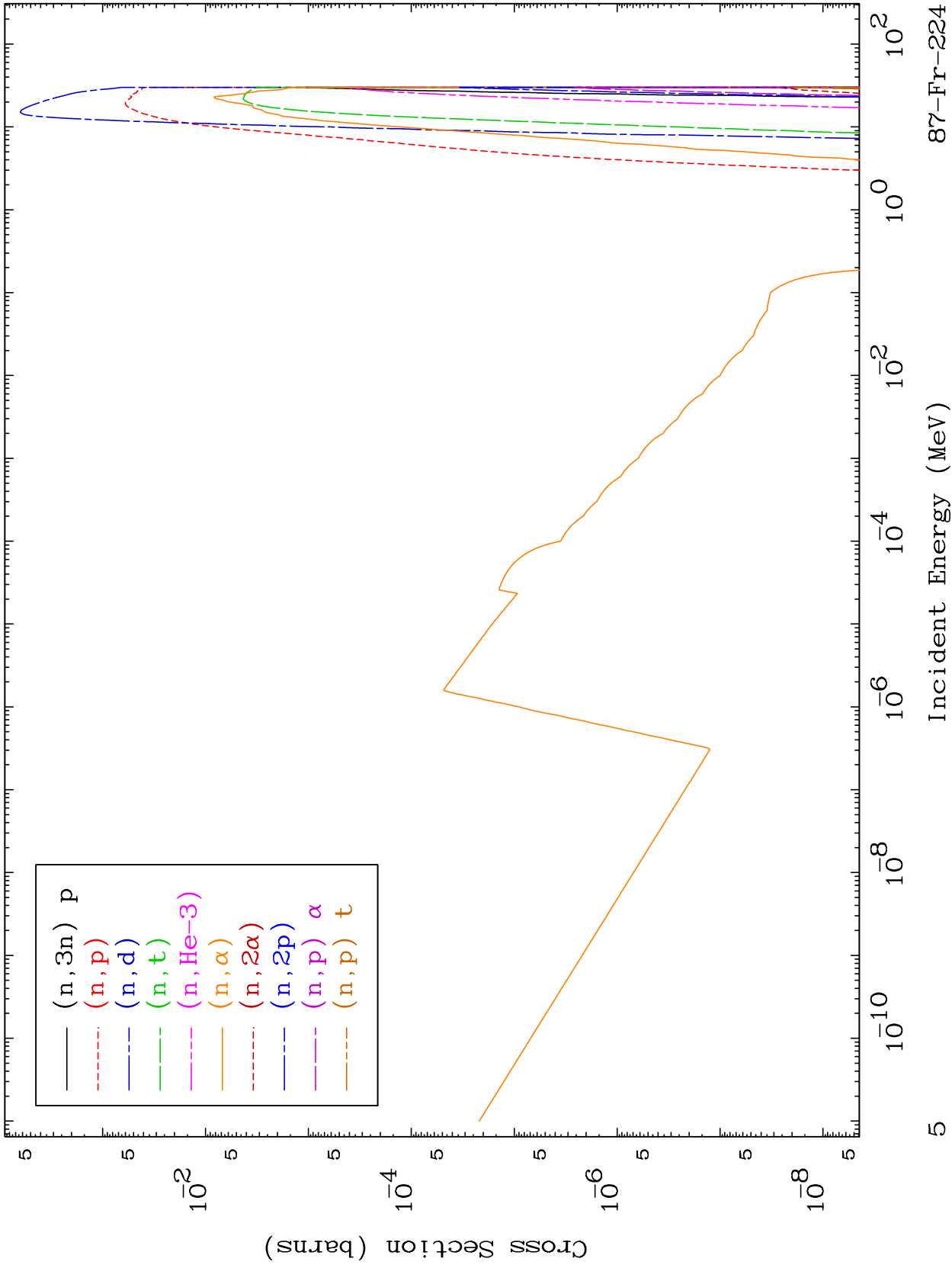


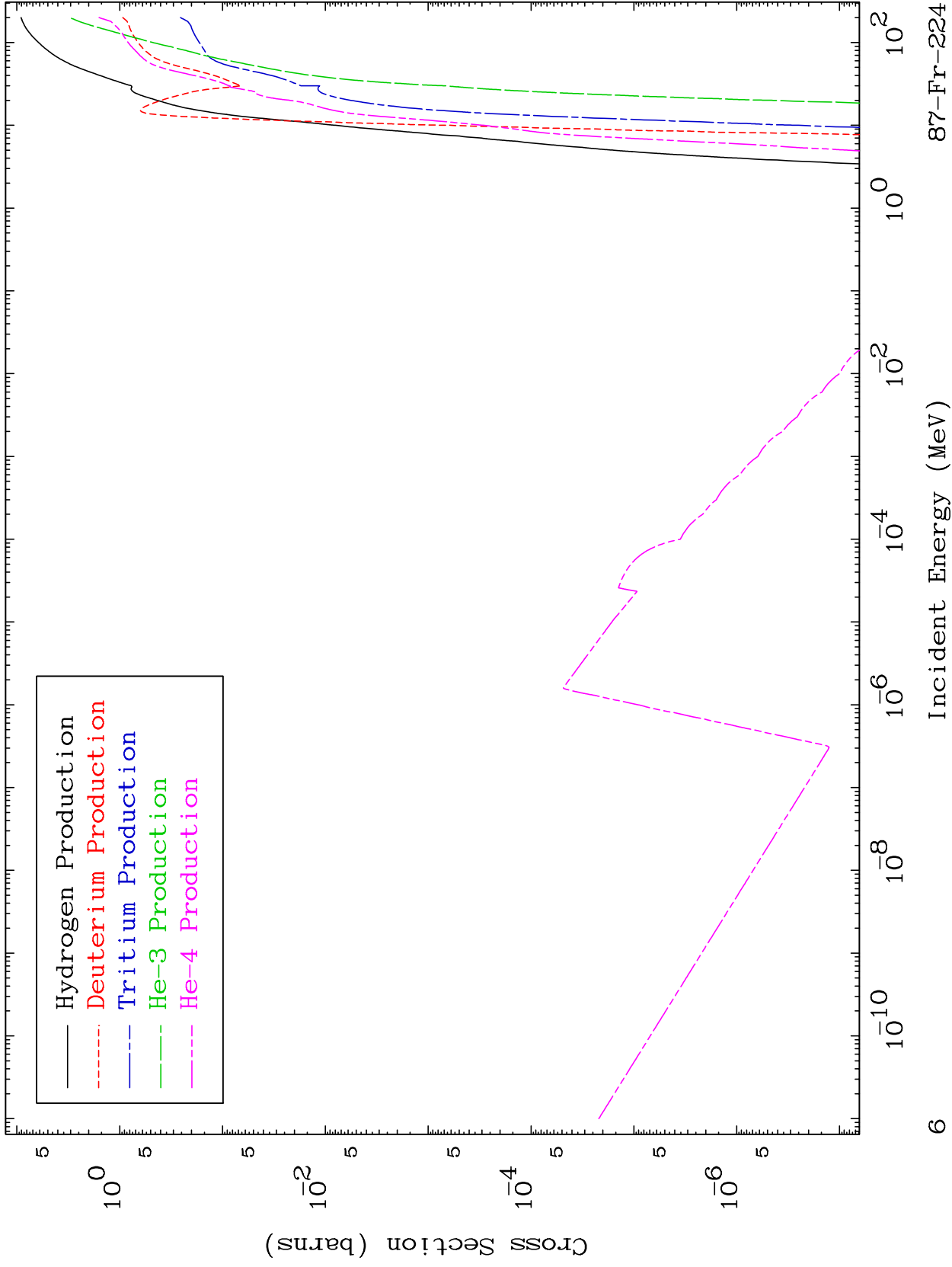


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Charged Particle
293 Kelvin Cross Sections

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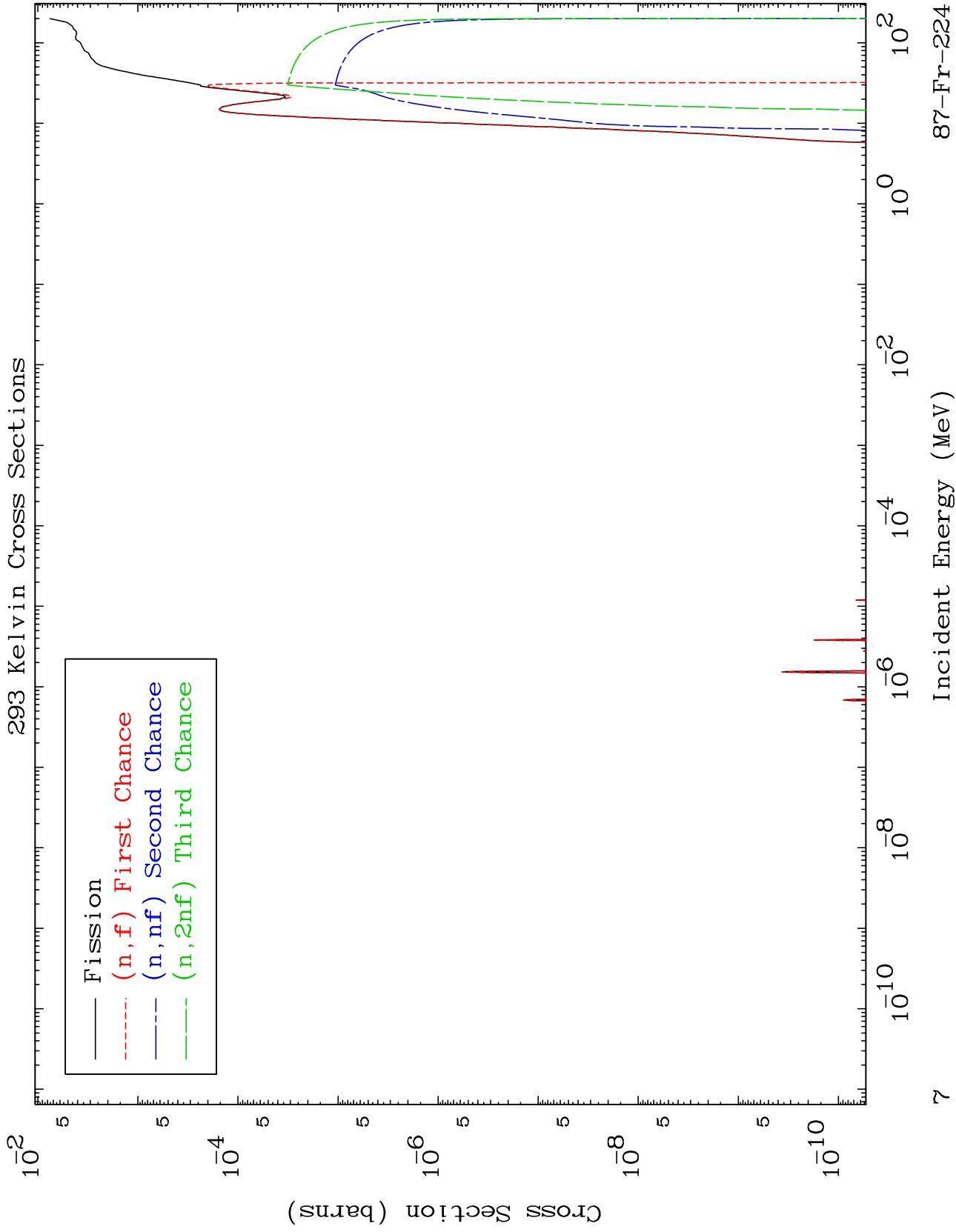


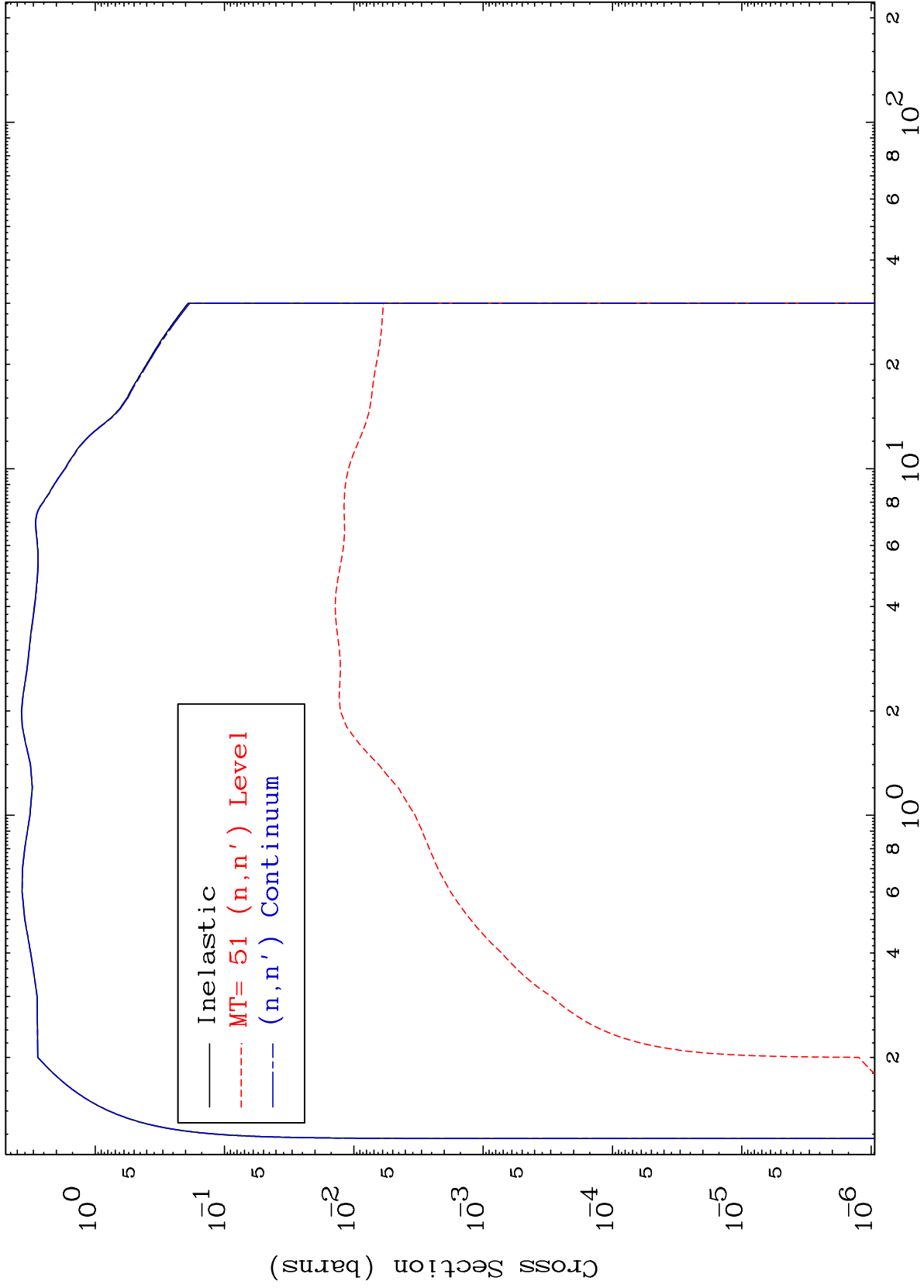


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Fission
293 Kelvin Cross Sections

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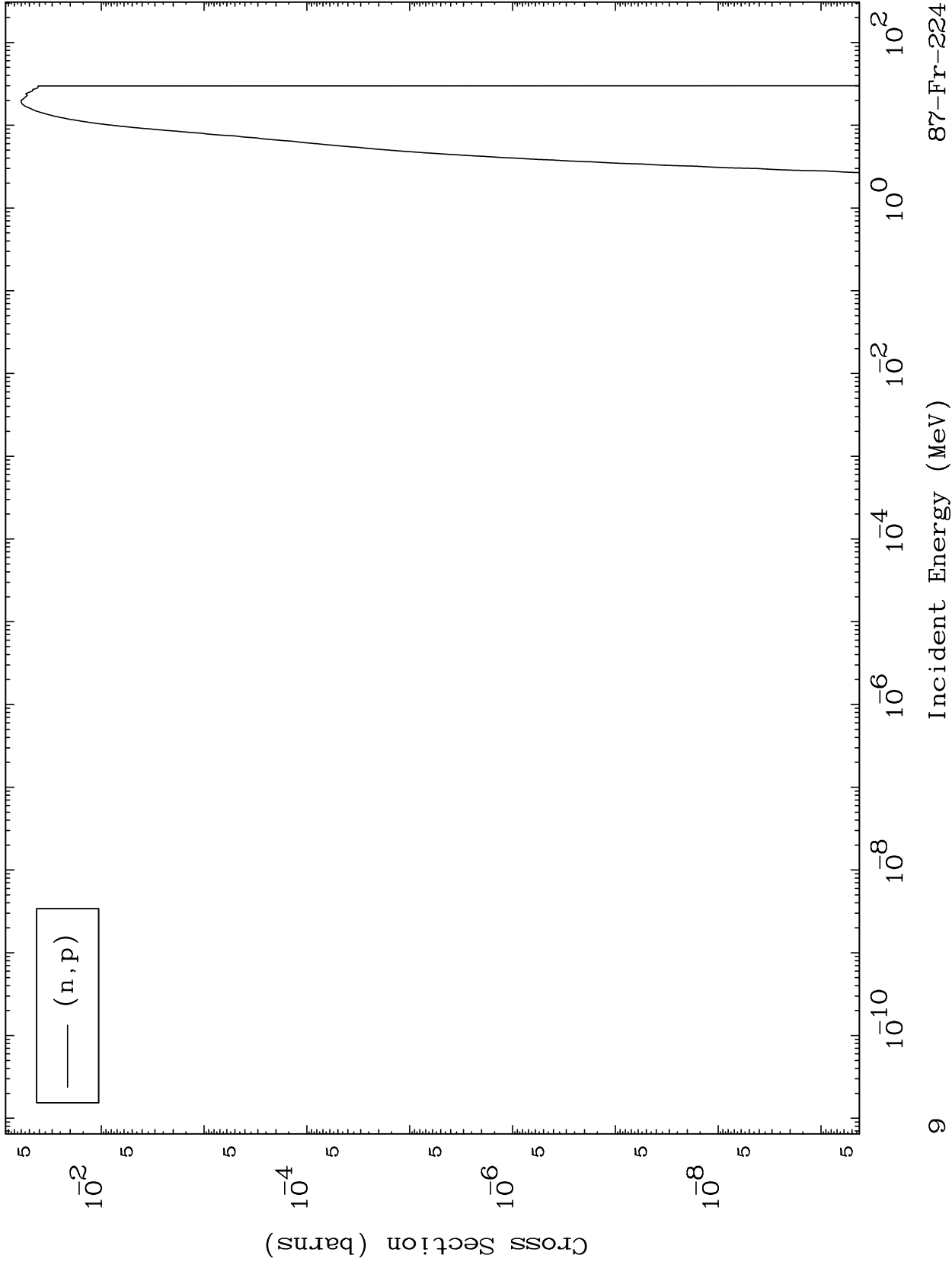




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(n,p) Levels
293 Kelvin Cross Sections

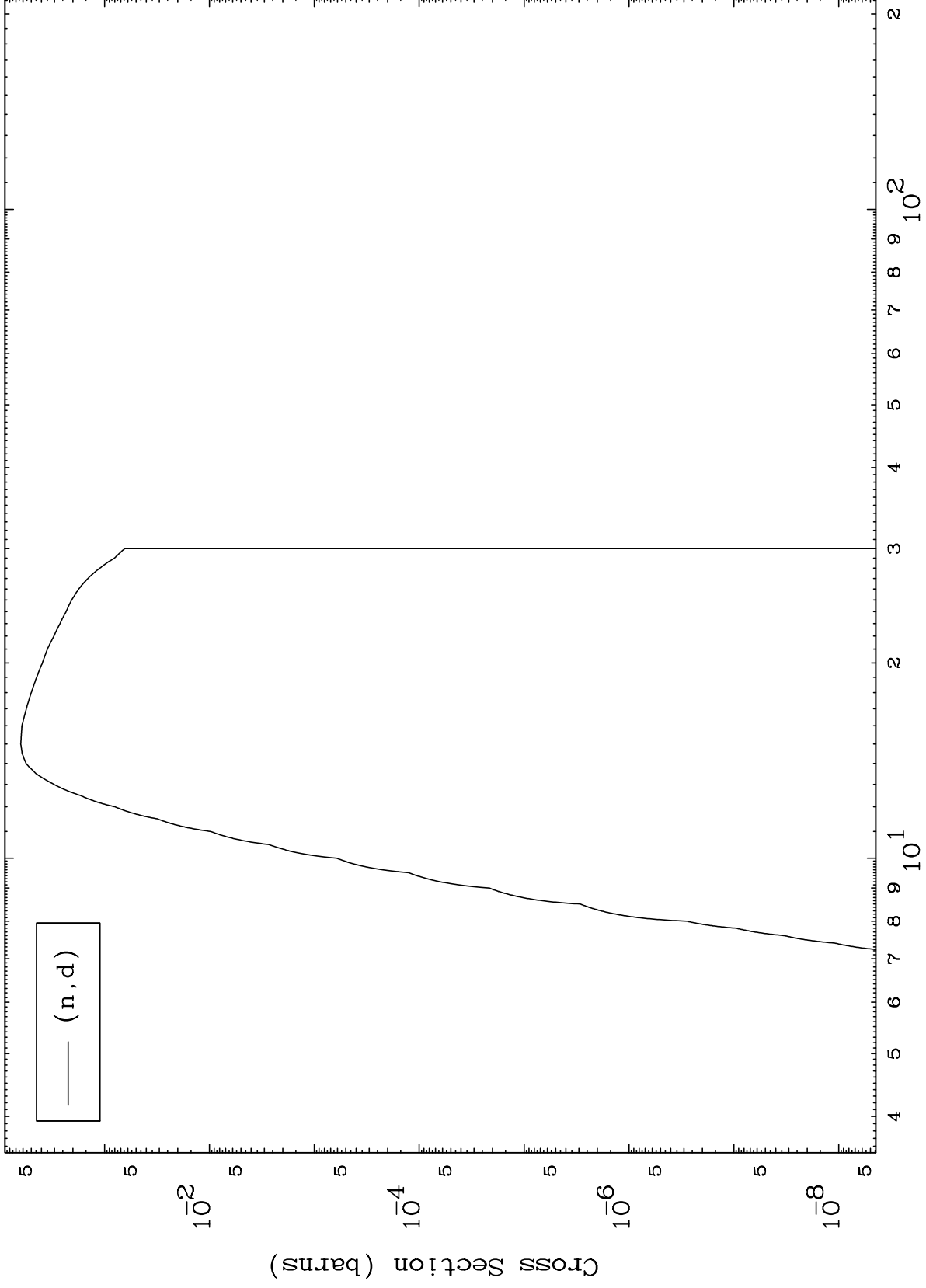
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(n,d) Levels
293 Kelvin Cross Sections

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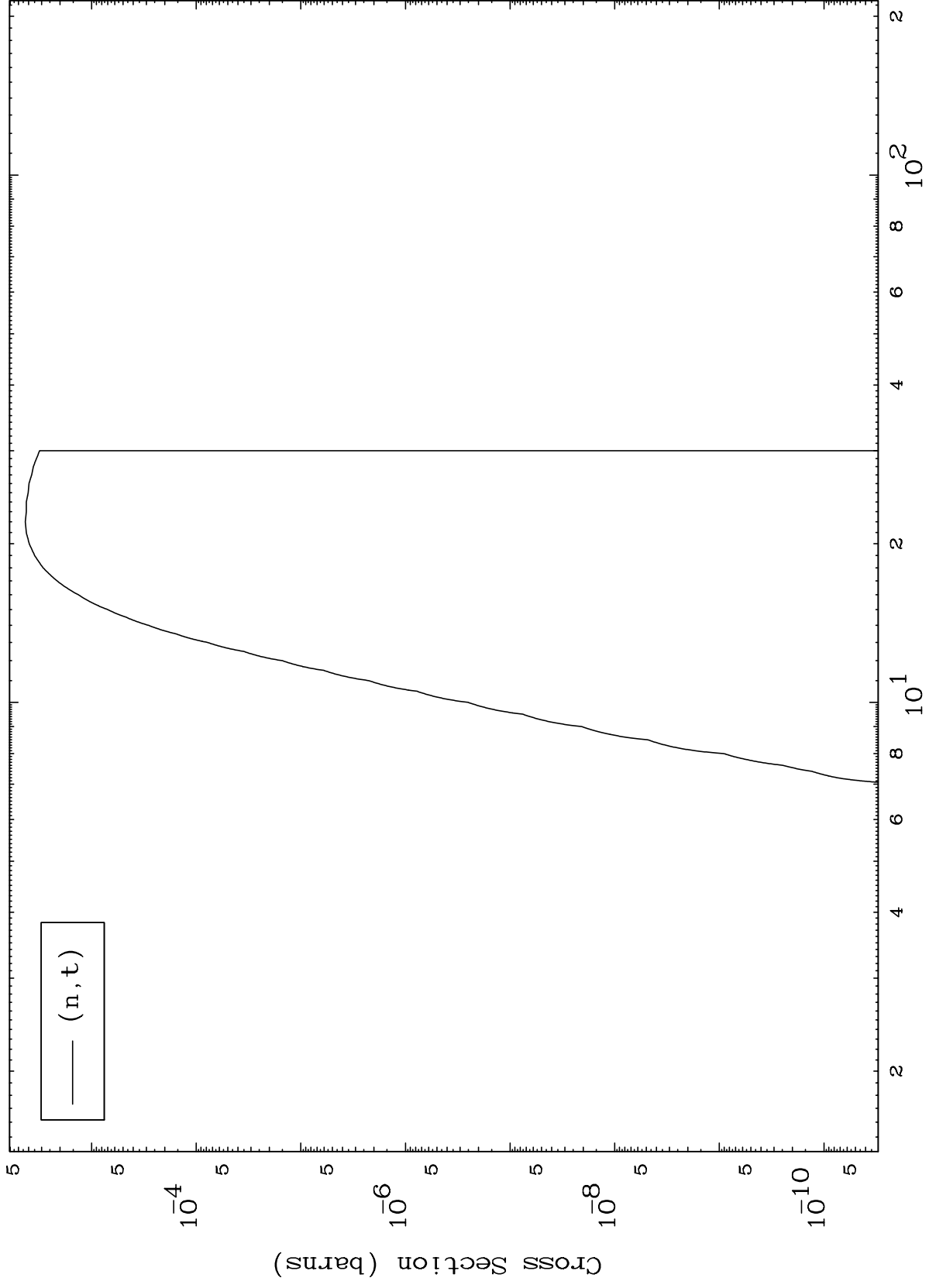
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

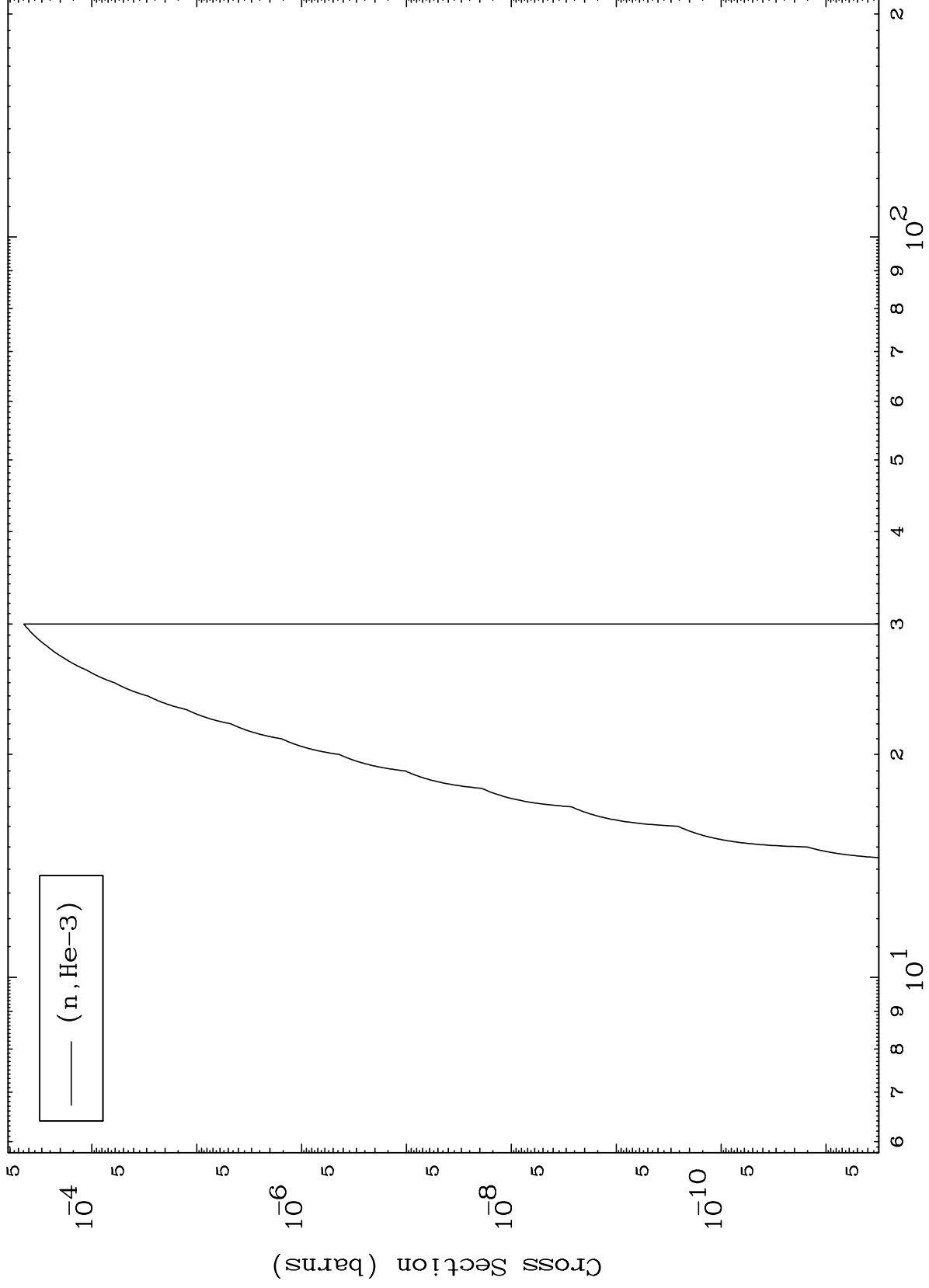
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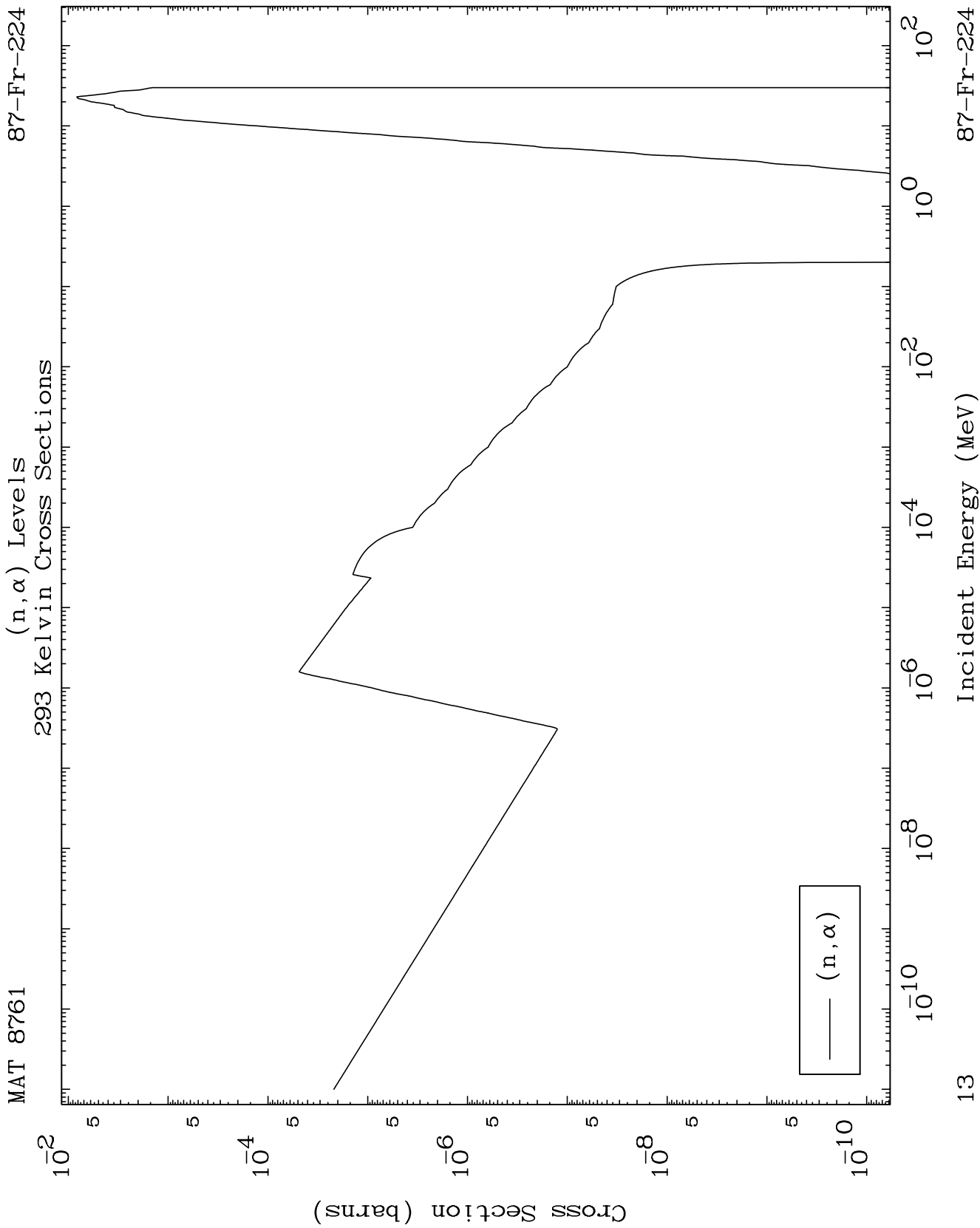
(n,He3) Levels
293 Kelvin Cross Sections

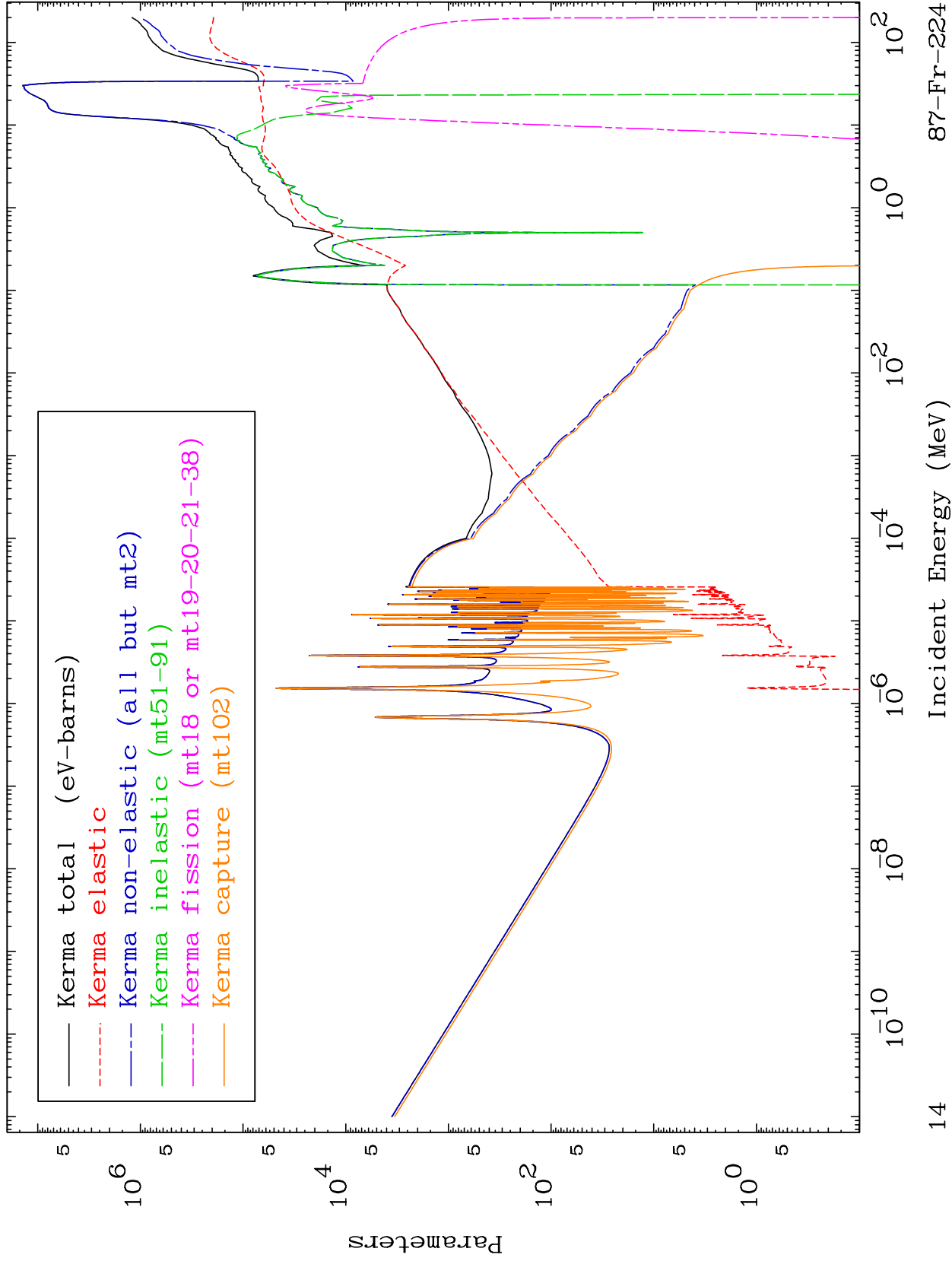
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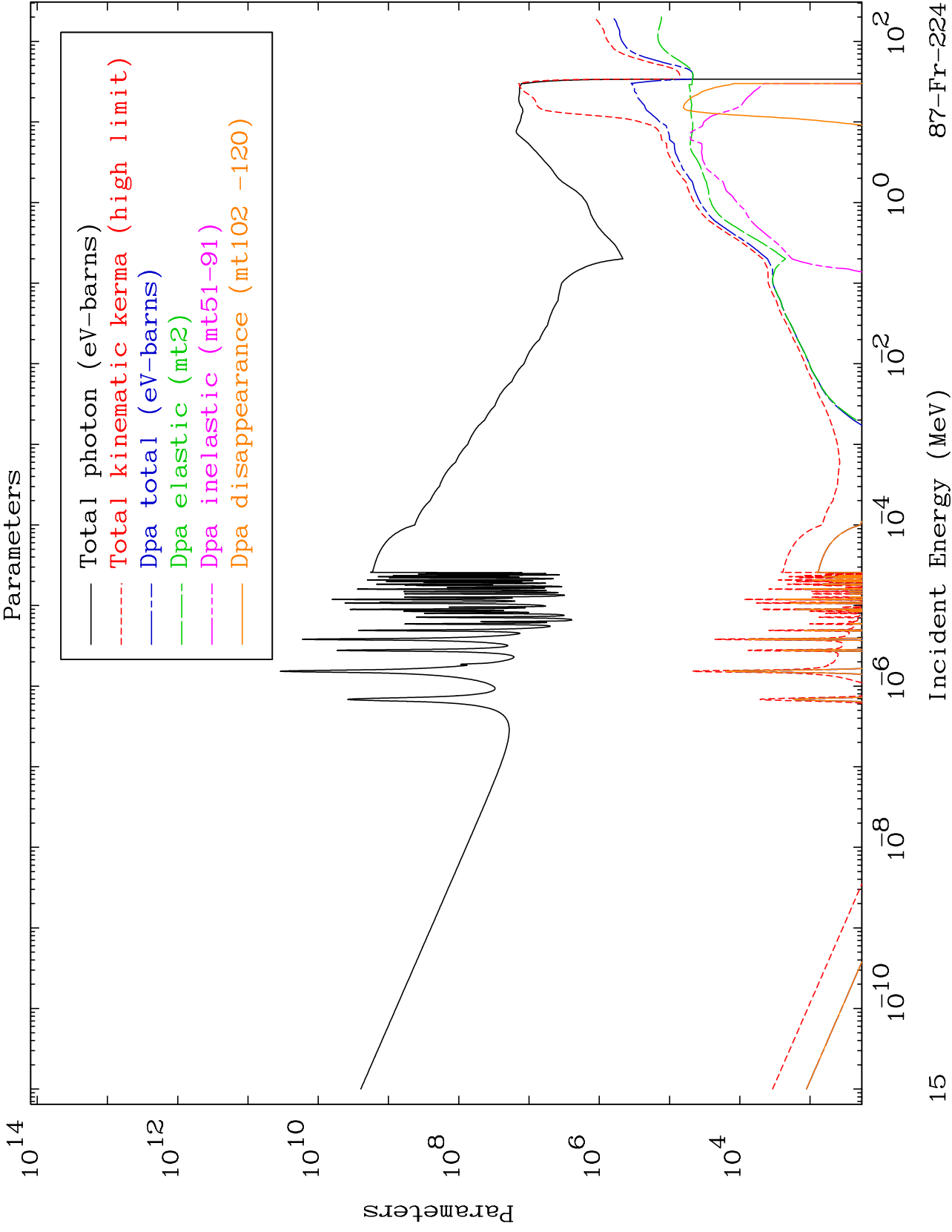


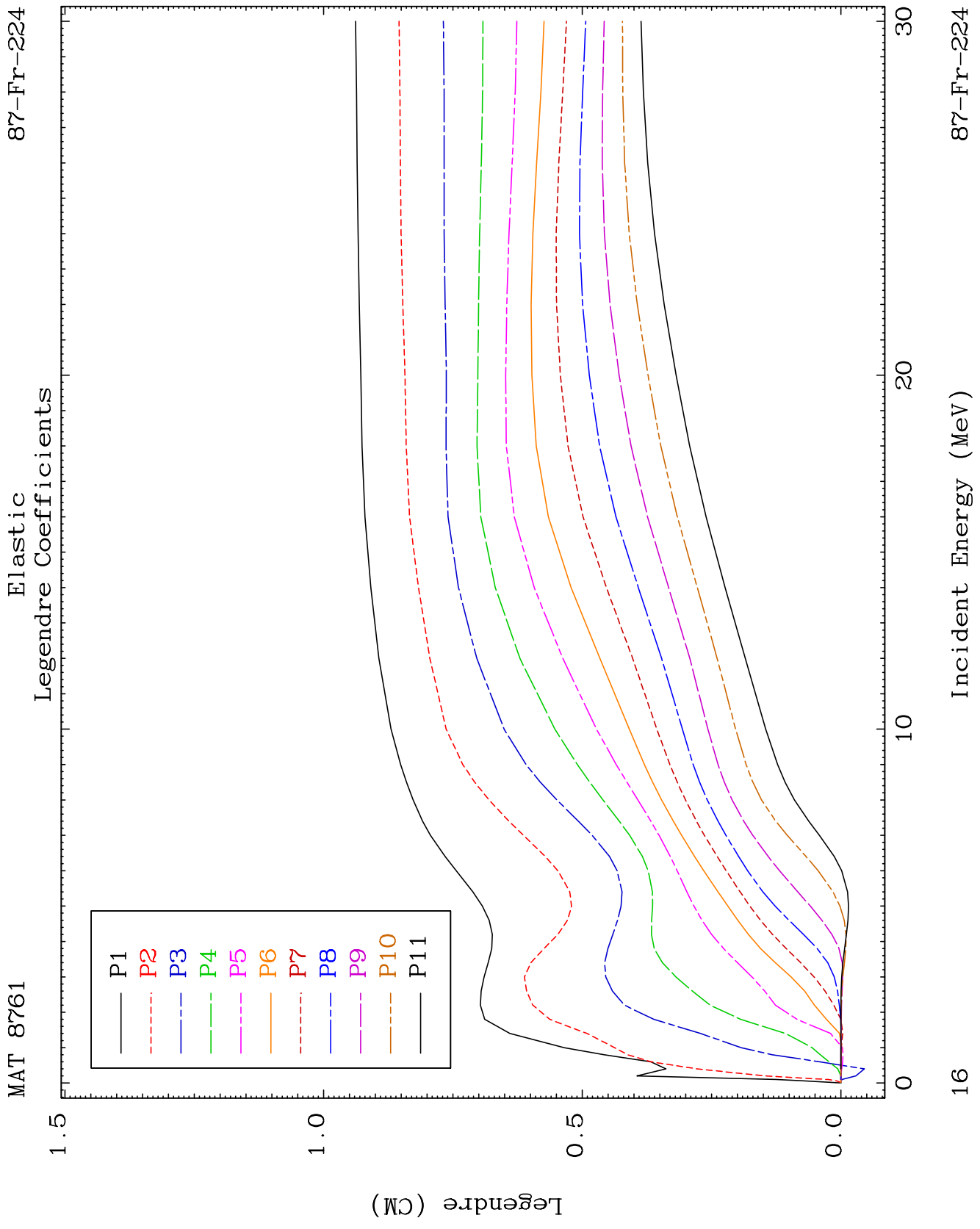
12

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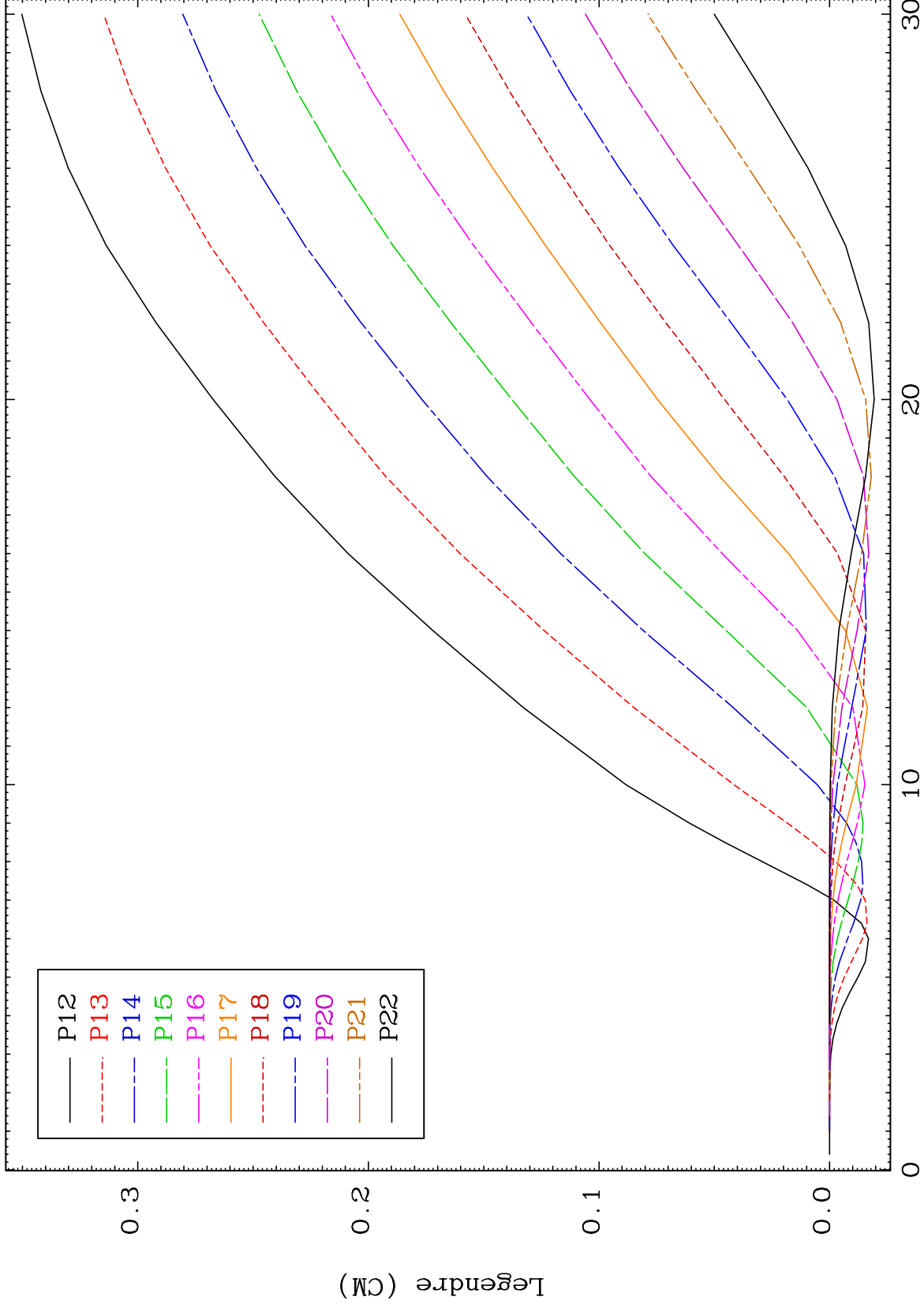




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Elastic Legendre Coefficients

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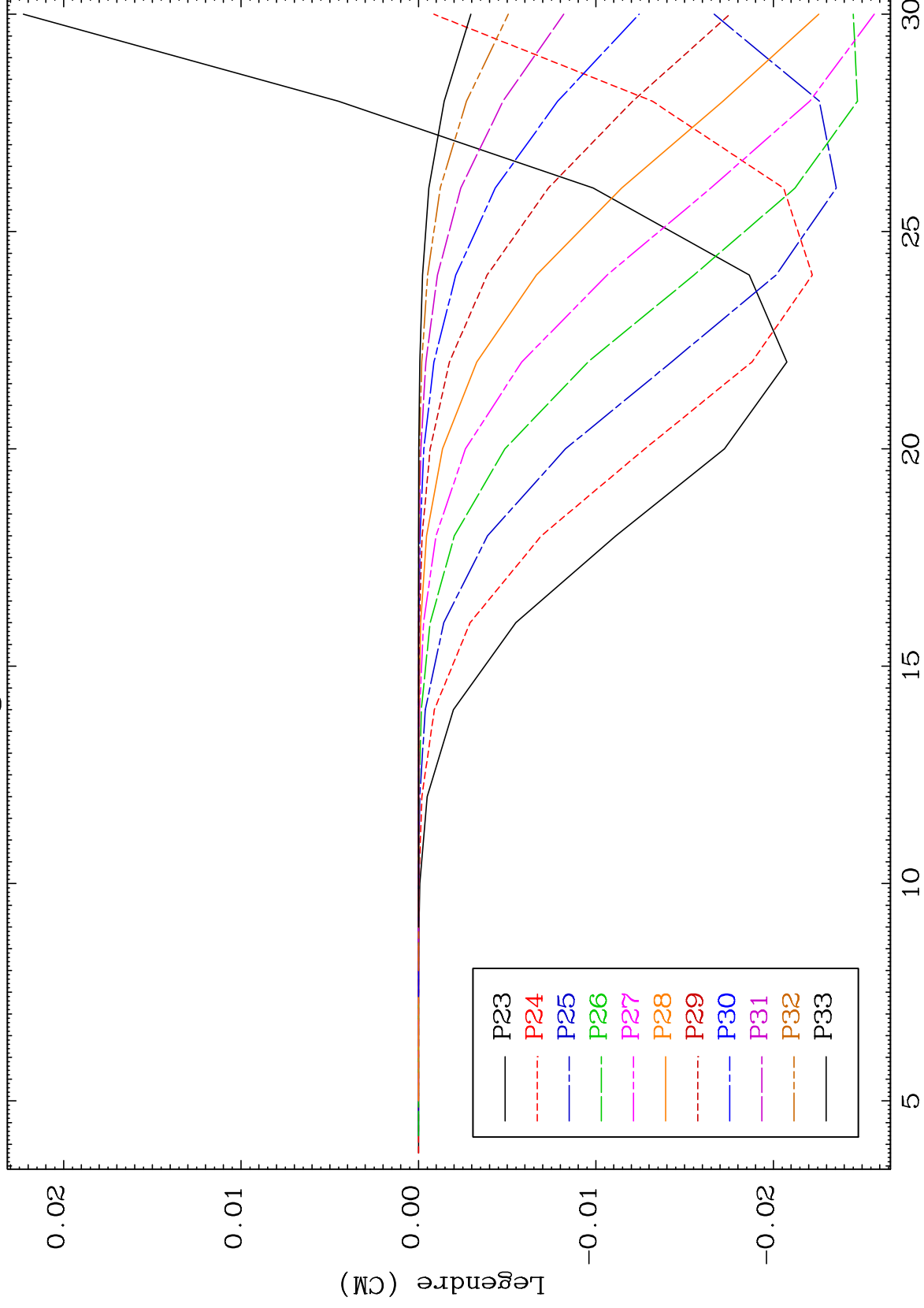
Incident Energy (MeV)

87-Fr-224

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Elastic Legendre Coefficients

87-Fr-224



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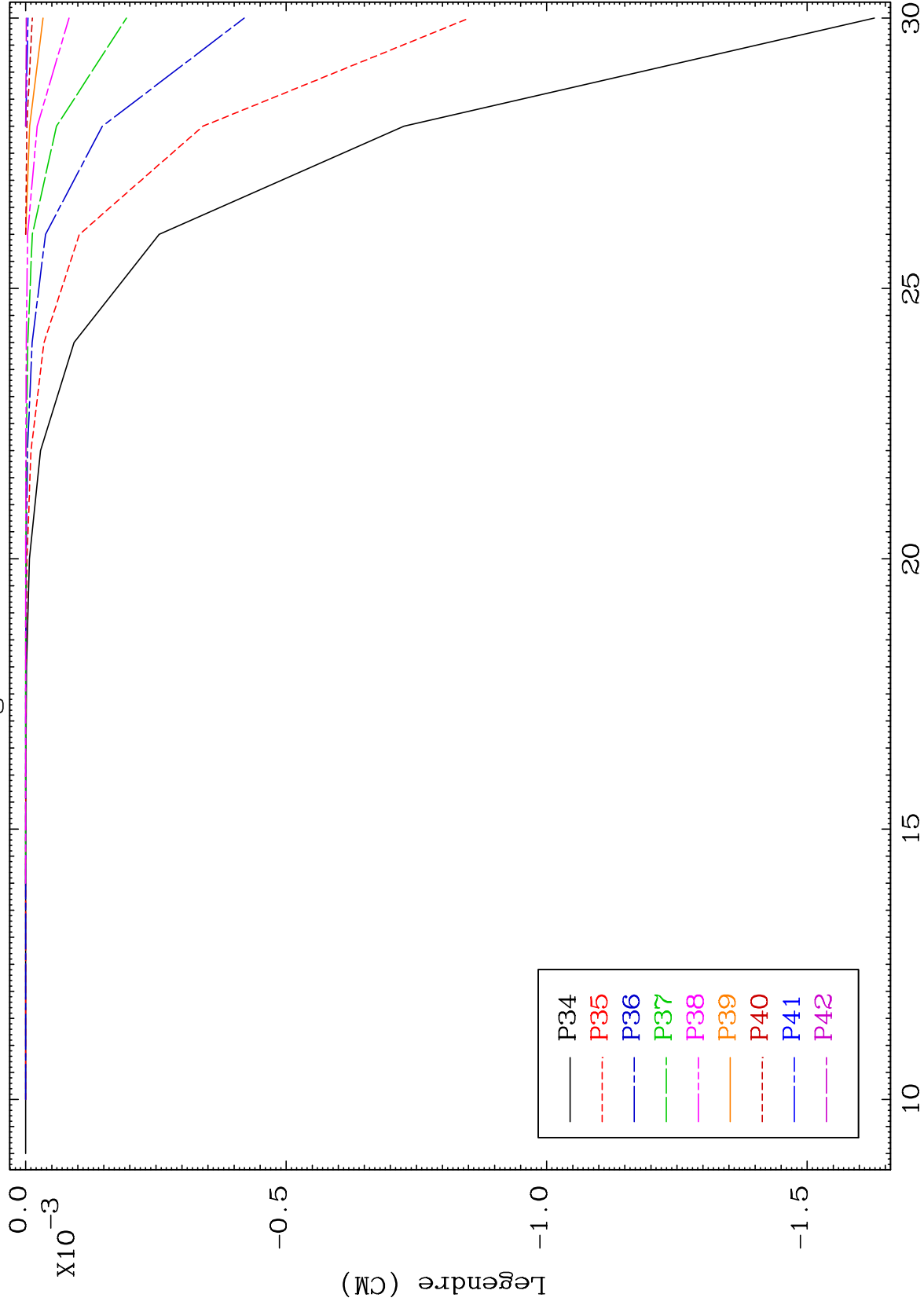
Incident Energy (MeV)

87-Fr-224

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Elastic Legendre Coefficients

87-Fr-224



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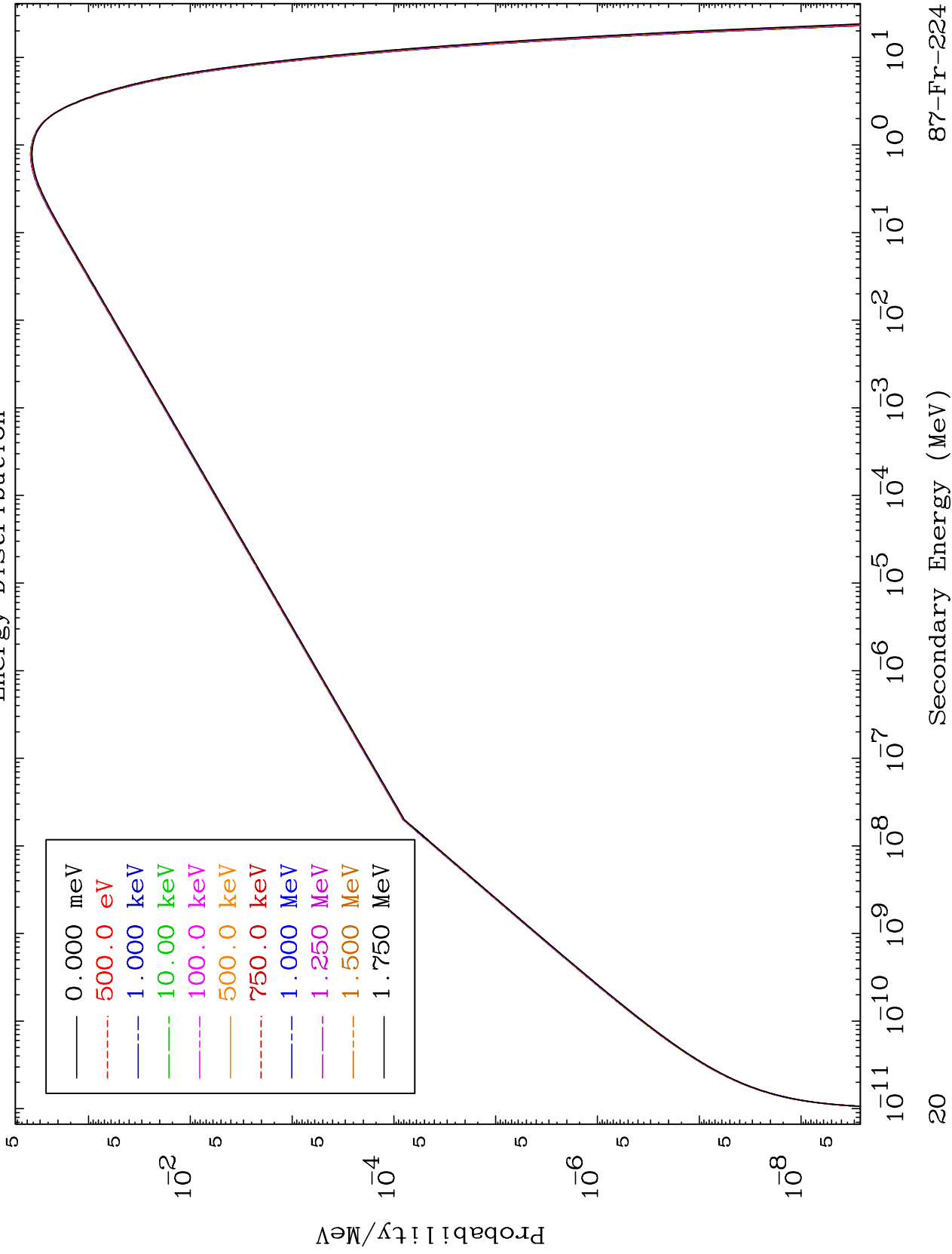
Incident Energy (MeV)

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Fission Energy Distribution

87-Fr-224



87-Fr-224

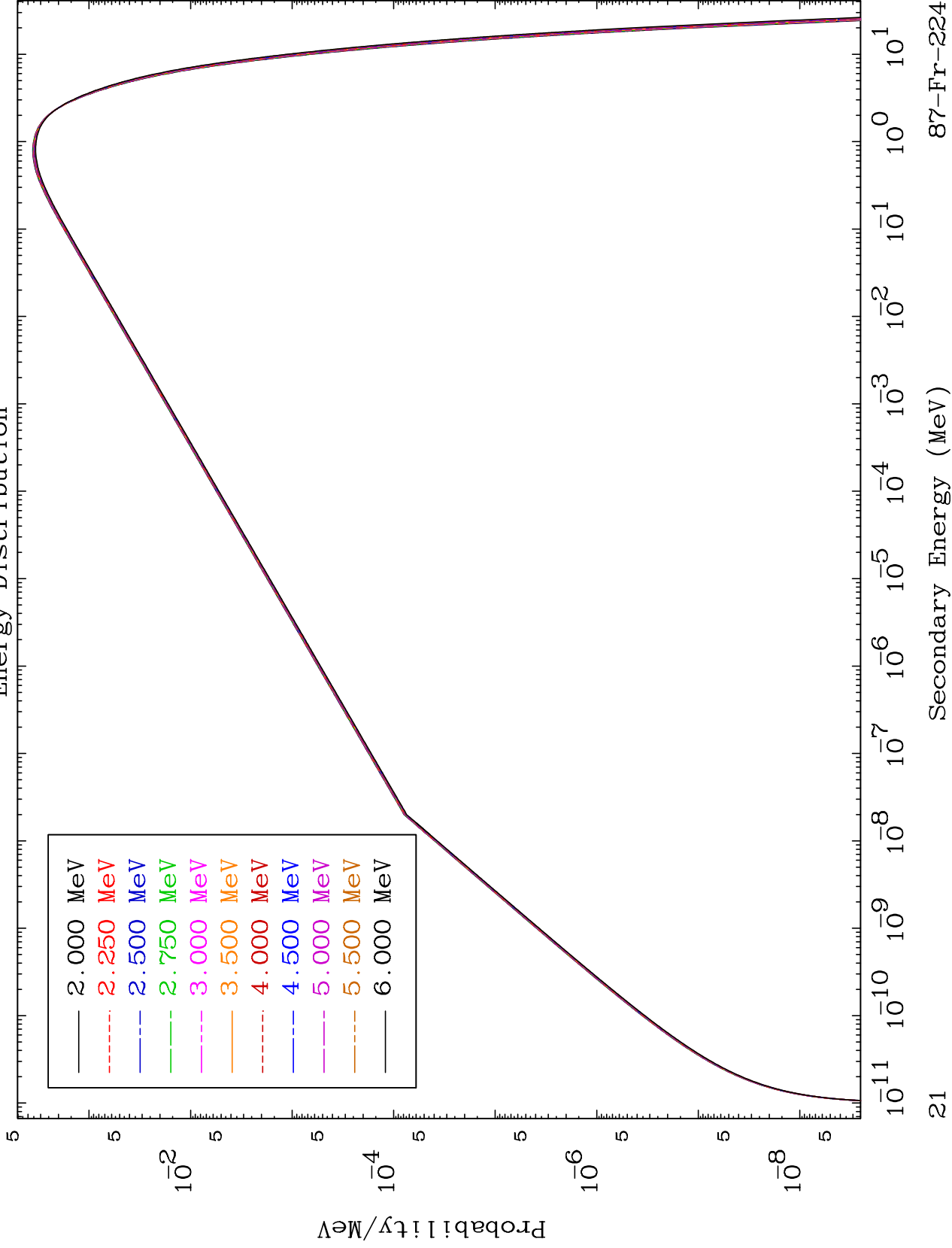
Secondary Energy (MeV)

20

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Fission Energy Distribution

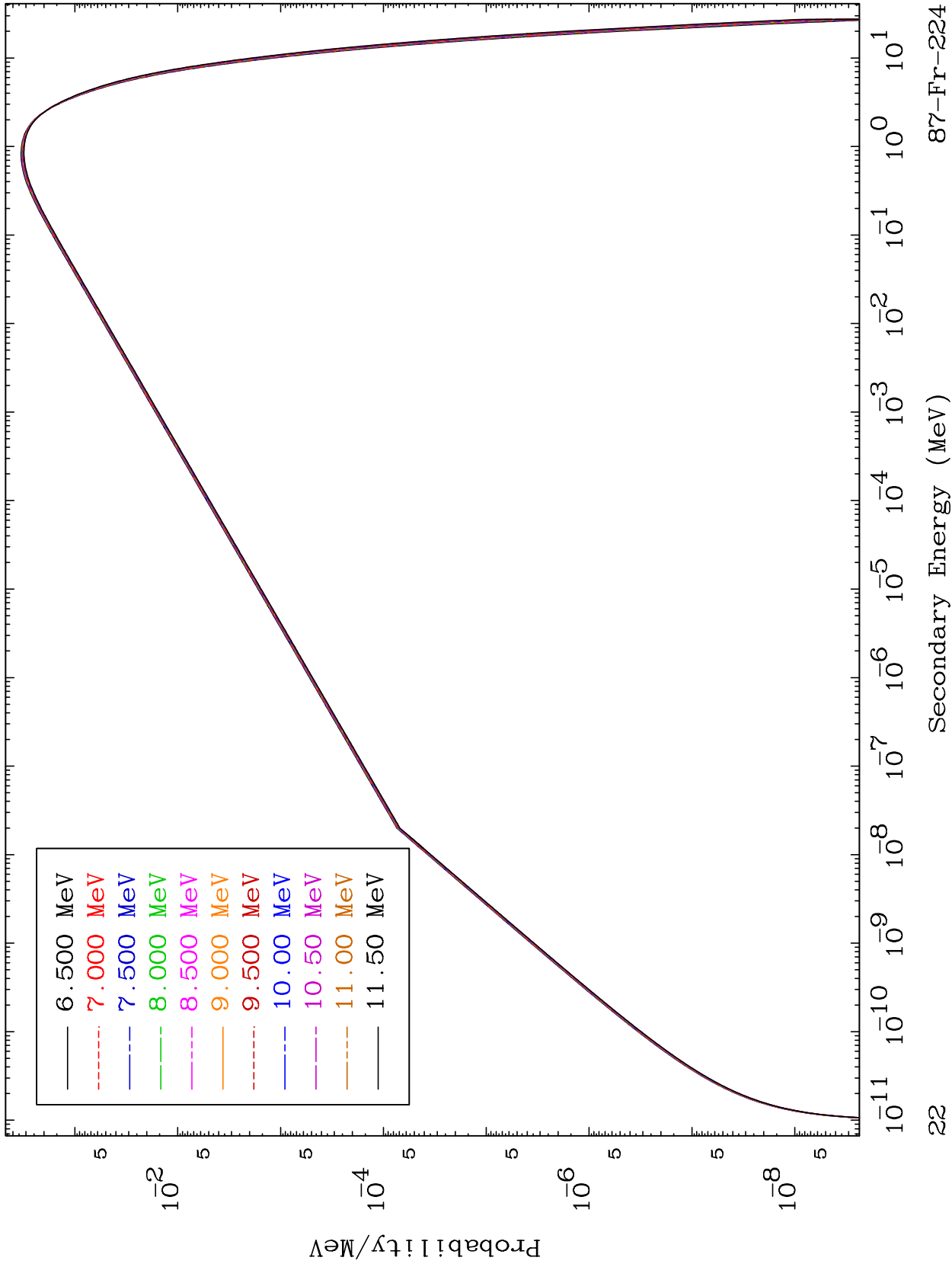
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Fission Energy Distribution

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87-Fr-224

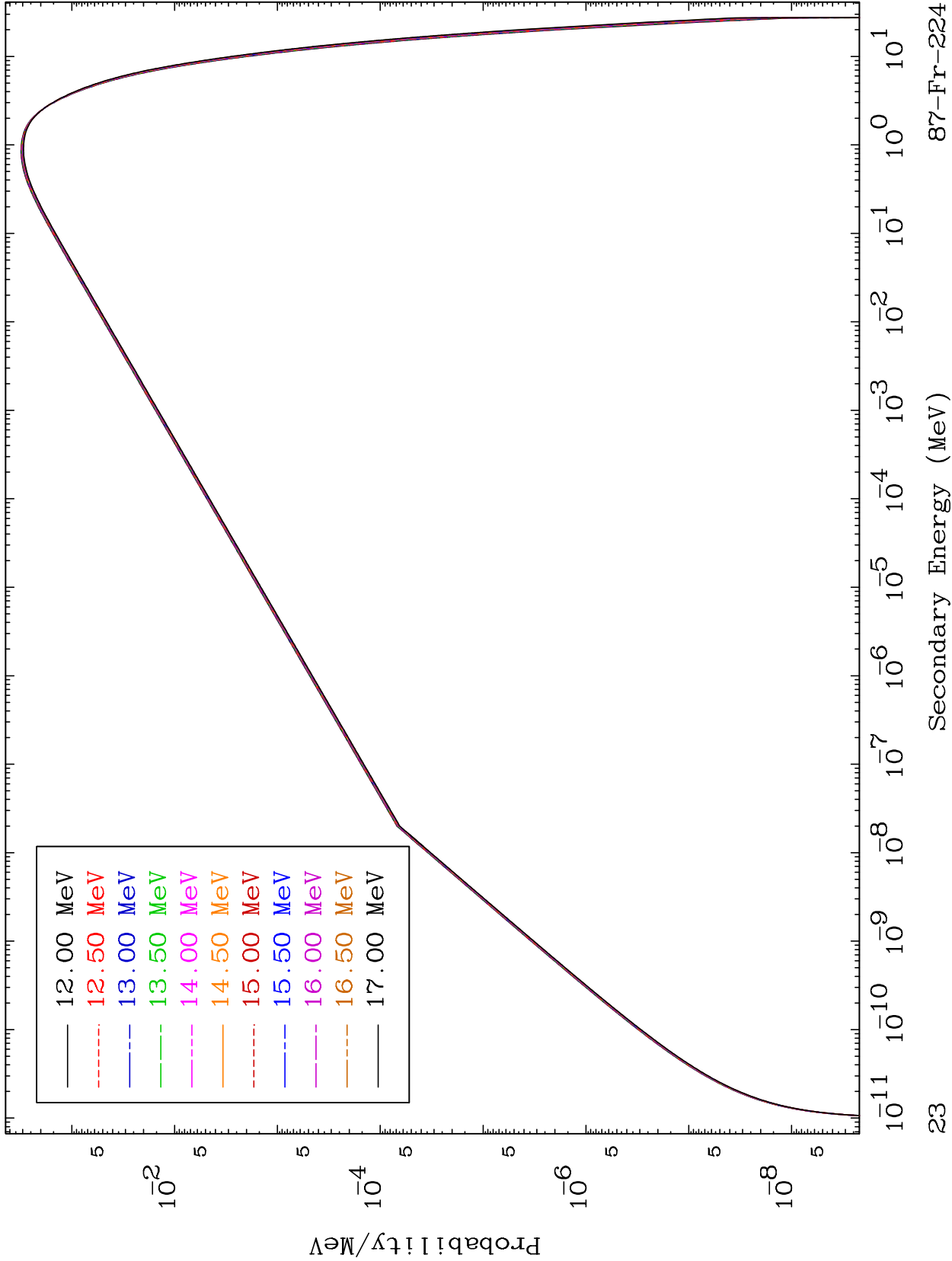
Secondary Energy (MeV)

22

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Fission Energy Distribution

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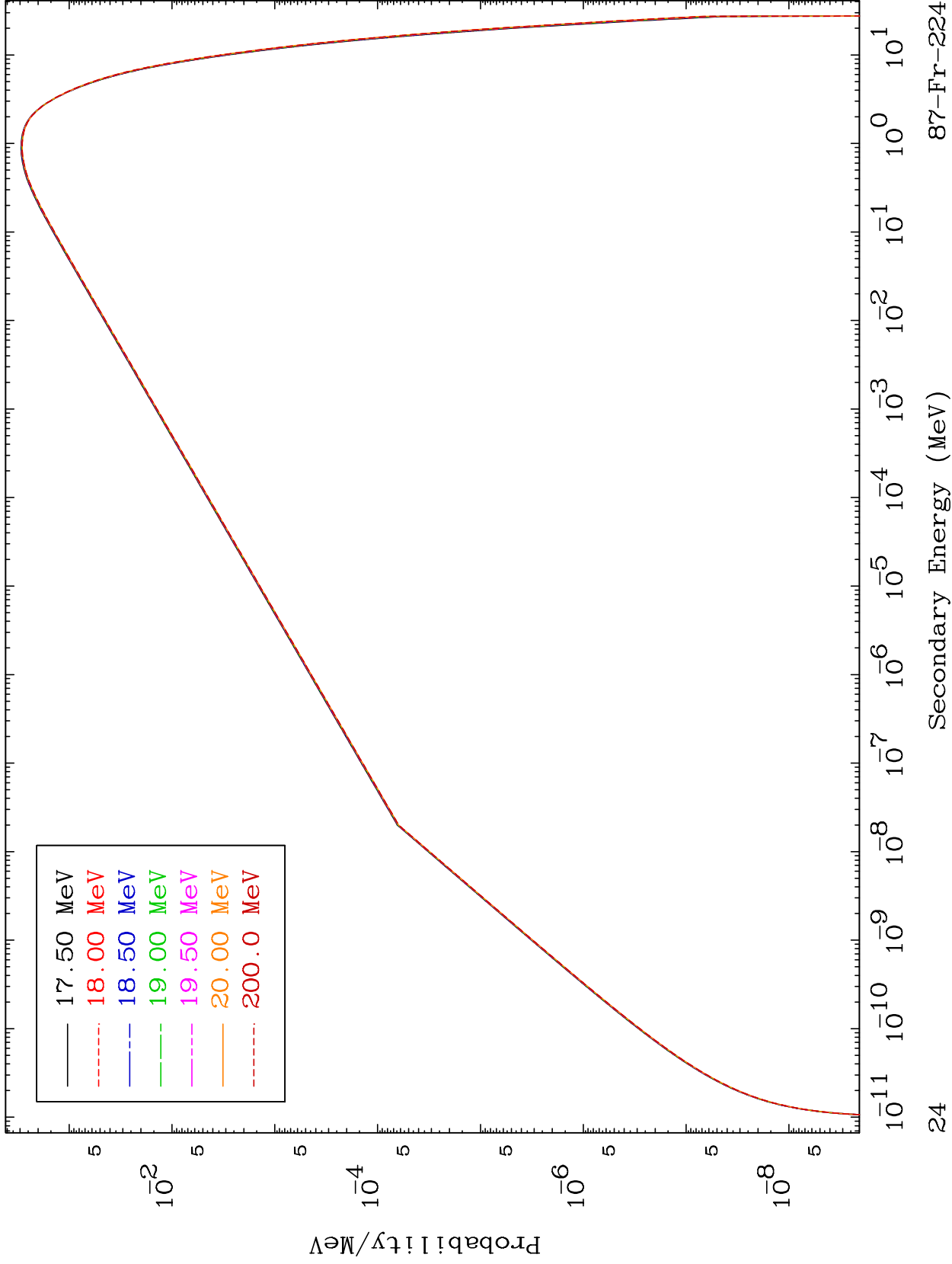
87-Fr-224

Secondary Energy (MeV)

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Fission Energy Distribution

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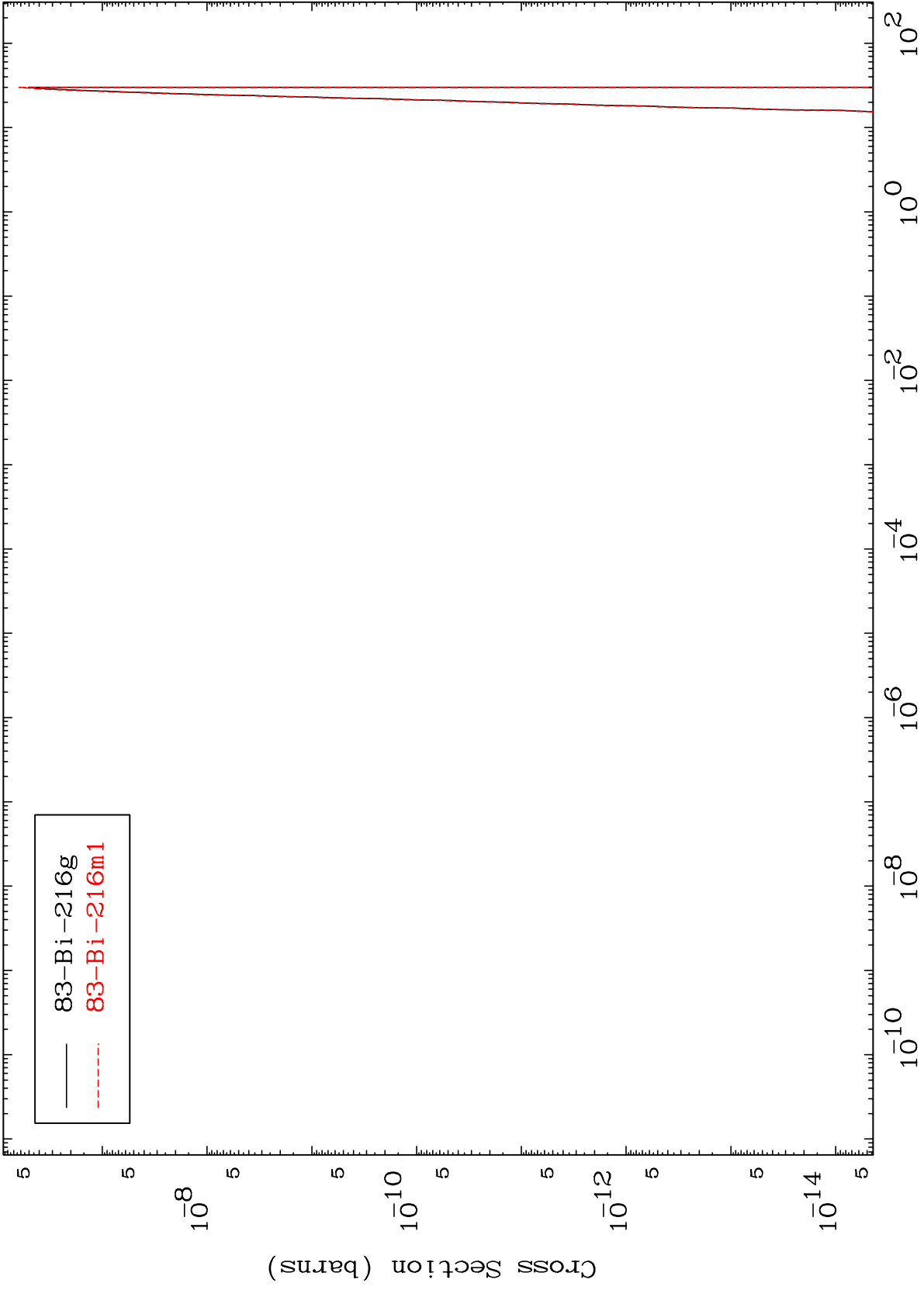
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(n,n') 2α

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

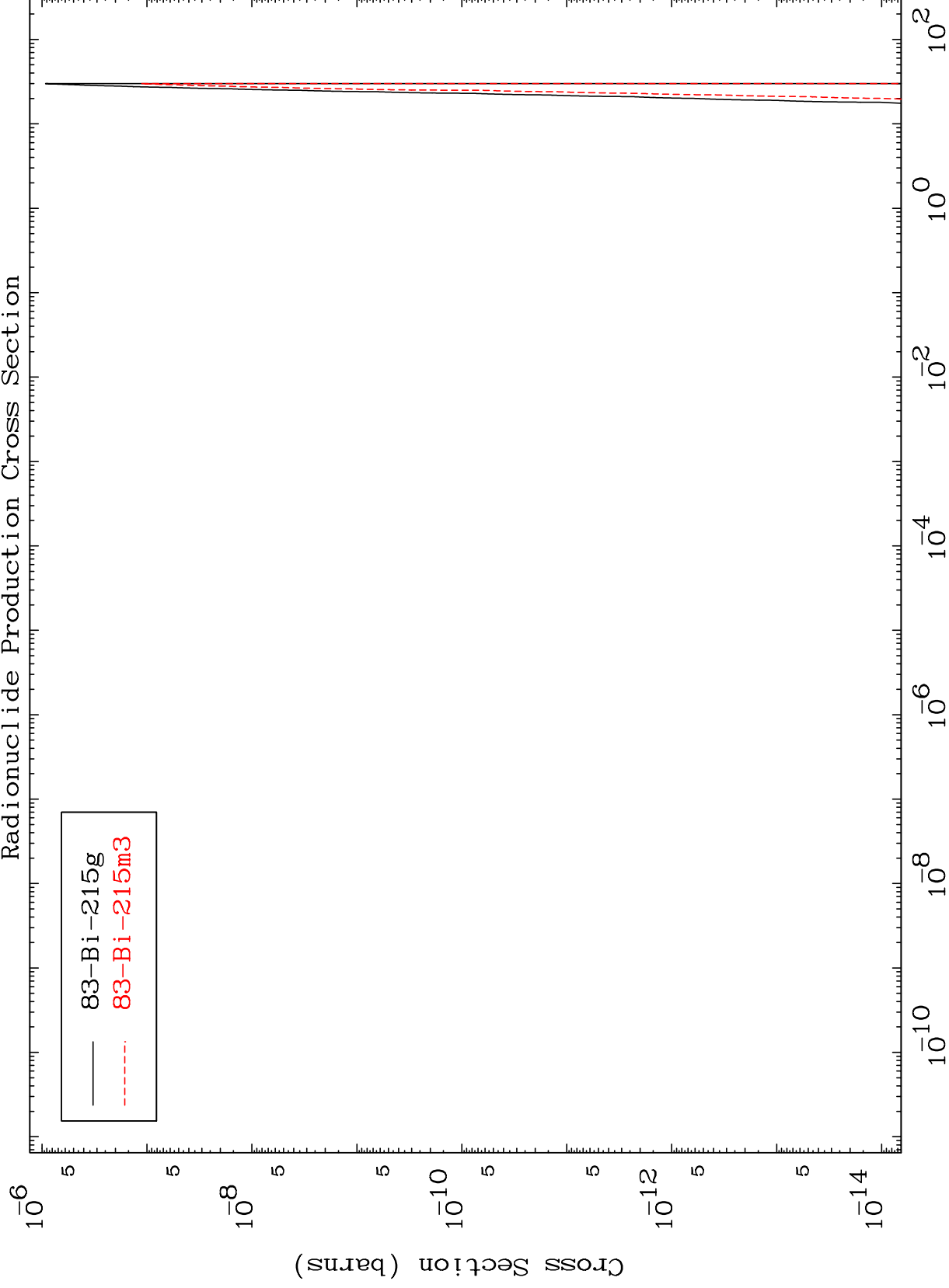
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(n,2n) 2 α

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Radionuclide Production Cross Section



Incident Energy (MeV)

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