

Program EVALPLOT  
(Version 2018-1)

by

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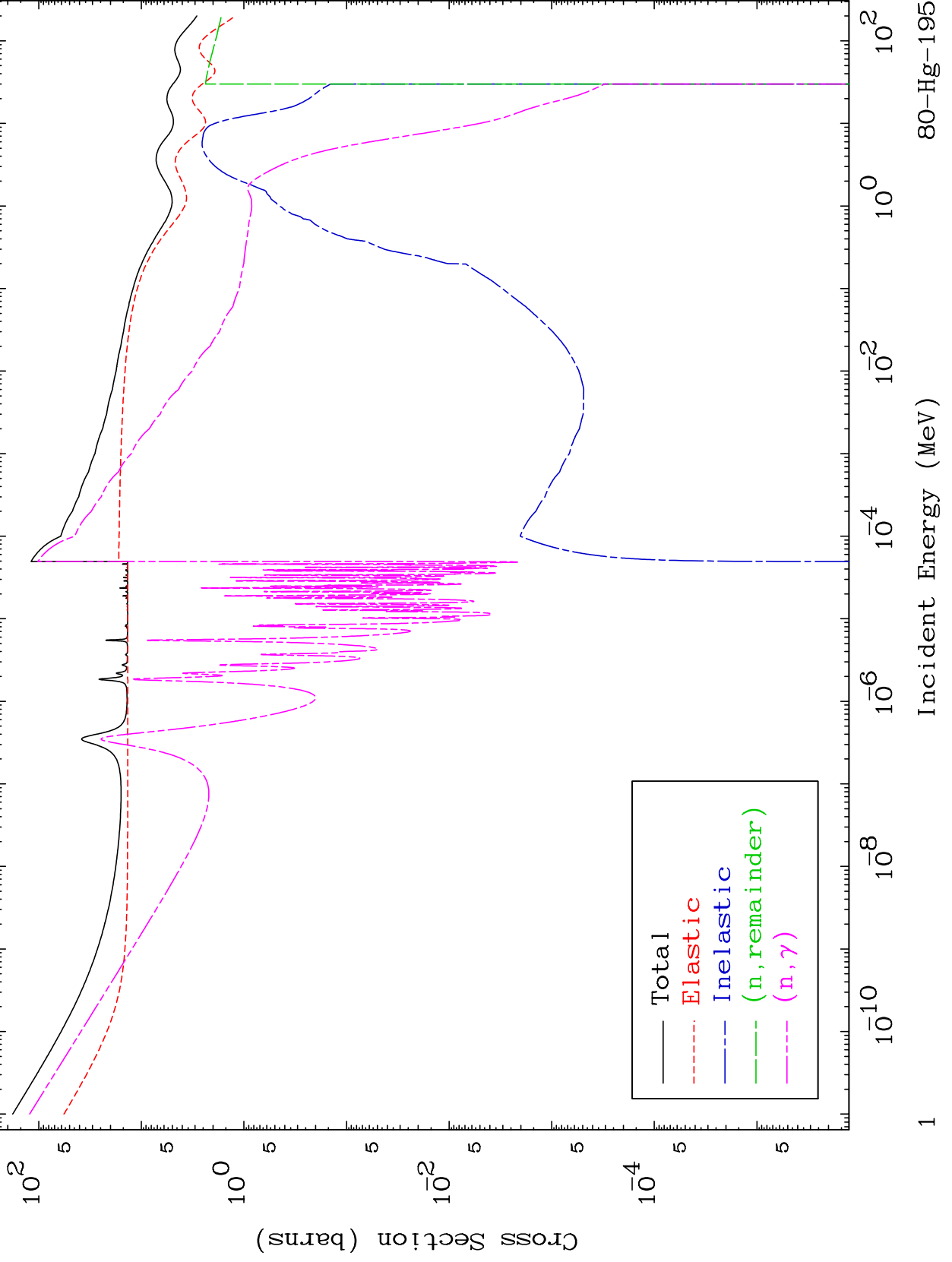
Press Mouse Button to Start

MAT 8023

Major

293 Kelvin Cross Sections

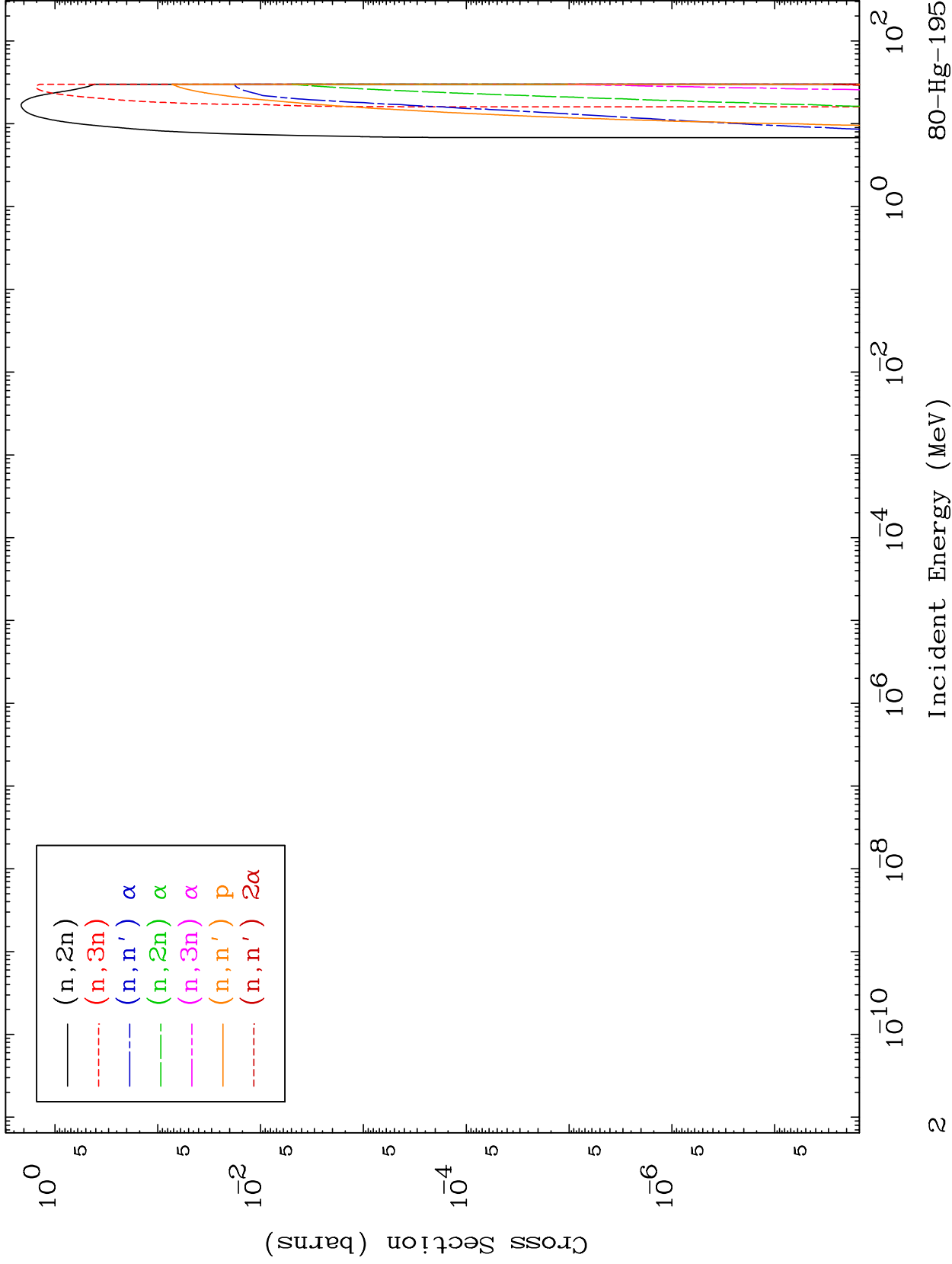
80-Hg-195



MAT 8023

Neutron Production  
293 Kelvin Cross Sections

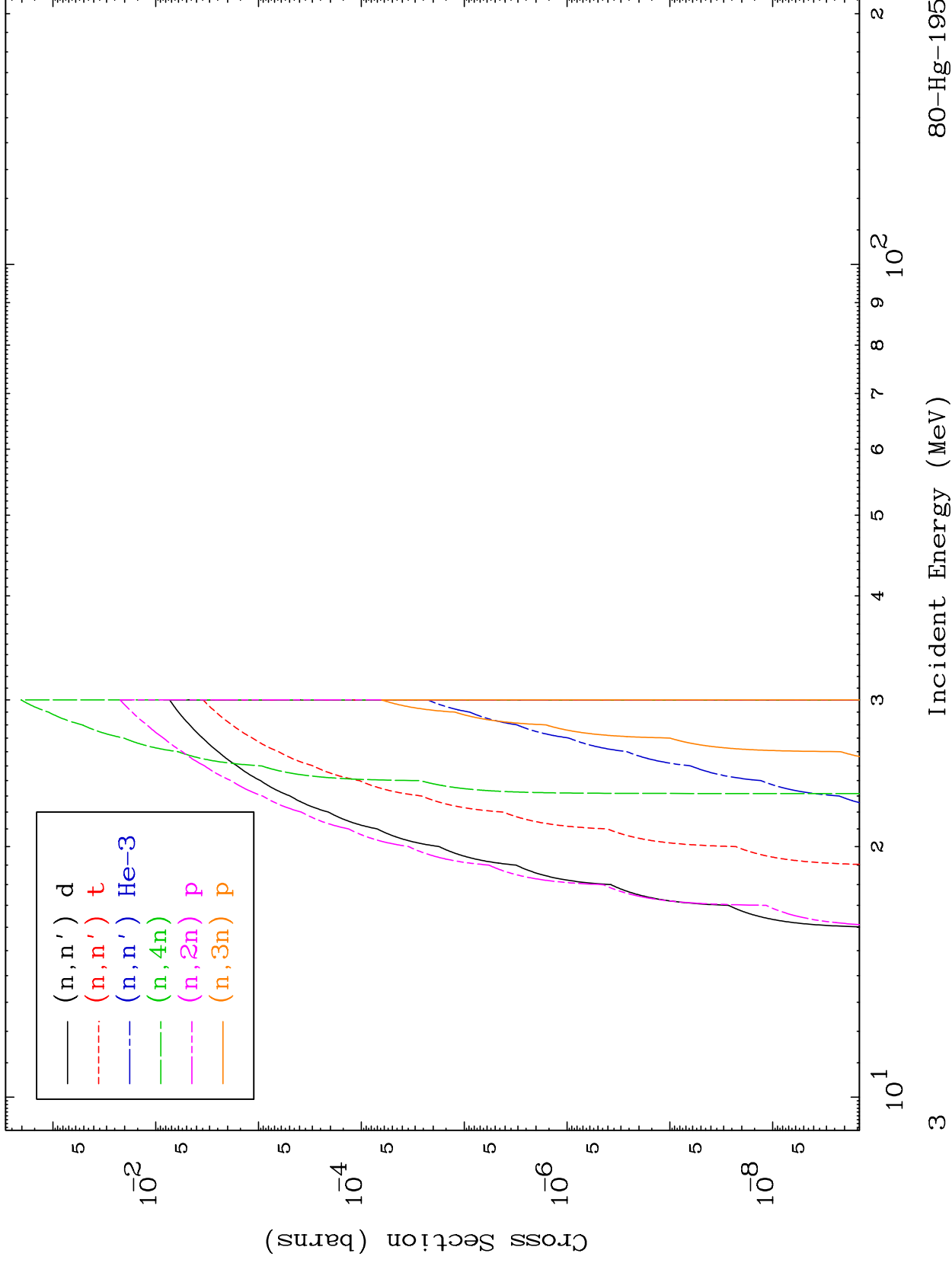
80-Hg-195



MAT 8023

Neutron Production  
293 Kelvin Cross Sections

80-Hg-195



80-Hg-195

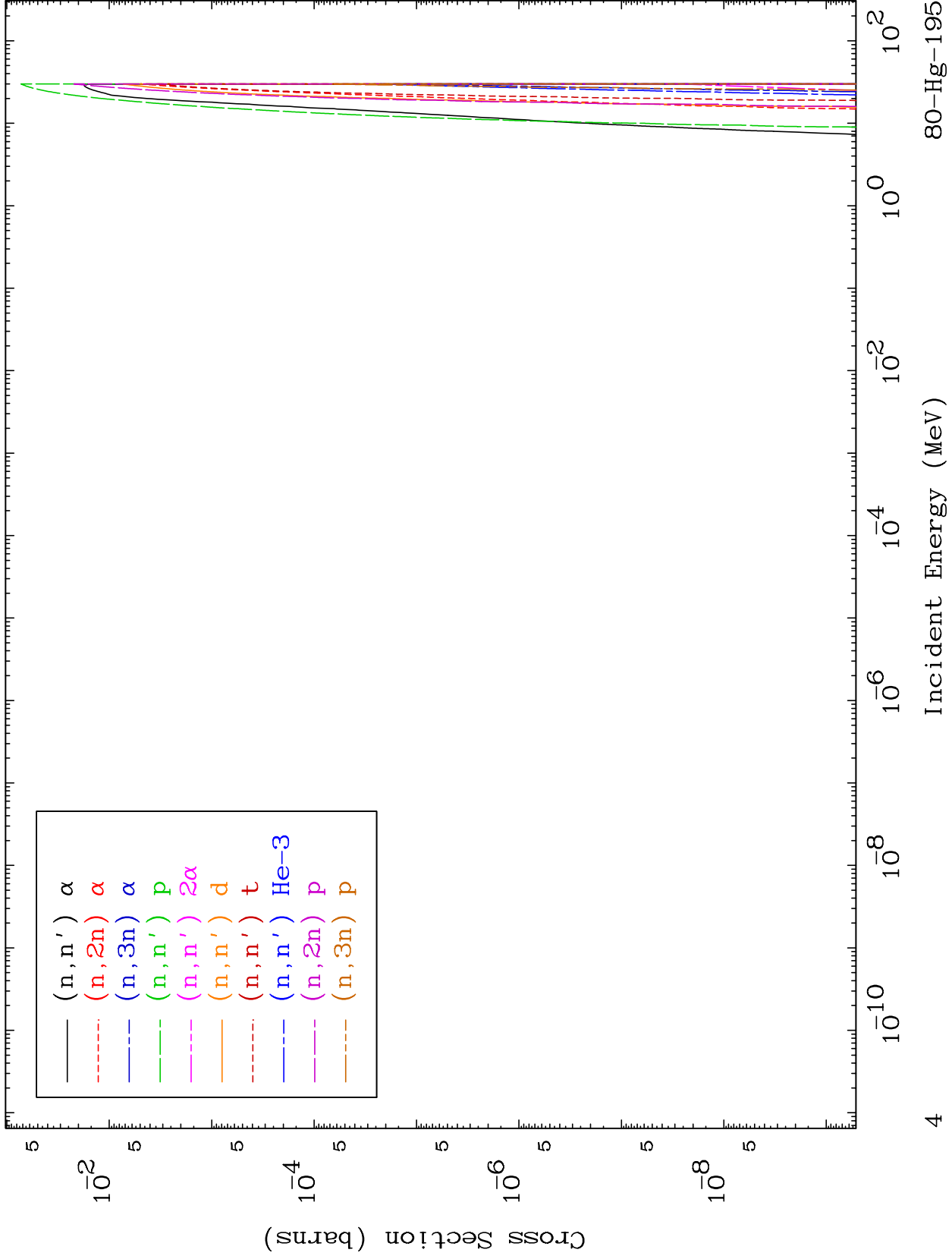
Incident Energy (MeV)

3

MAT 8023

Charged Particle  
293 Kelvin Cross Sections

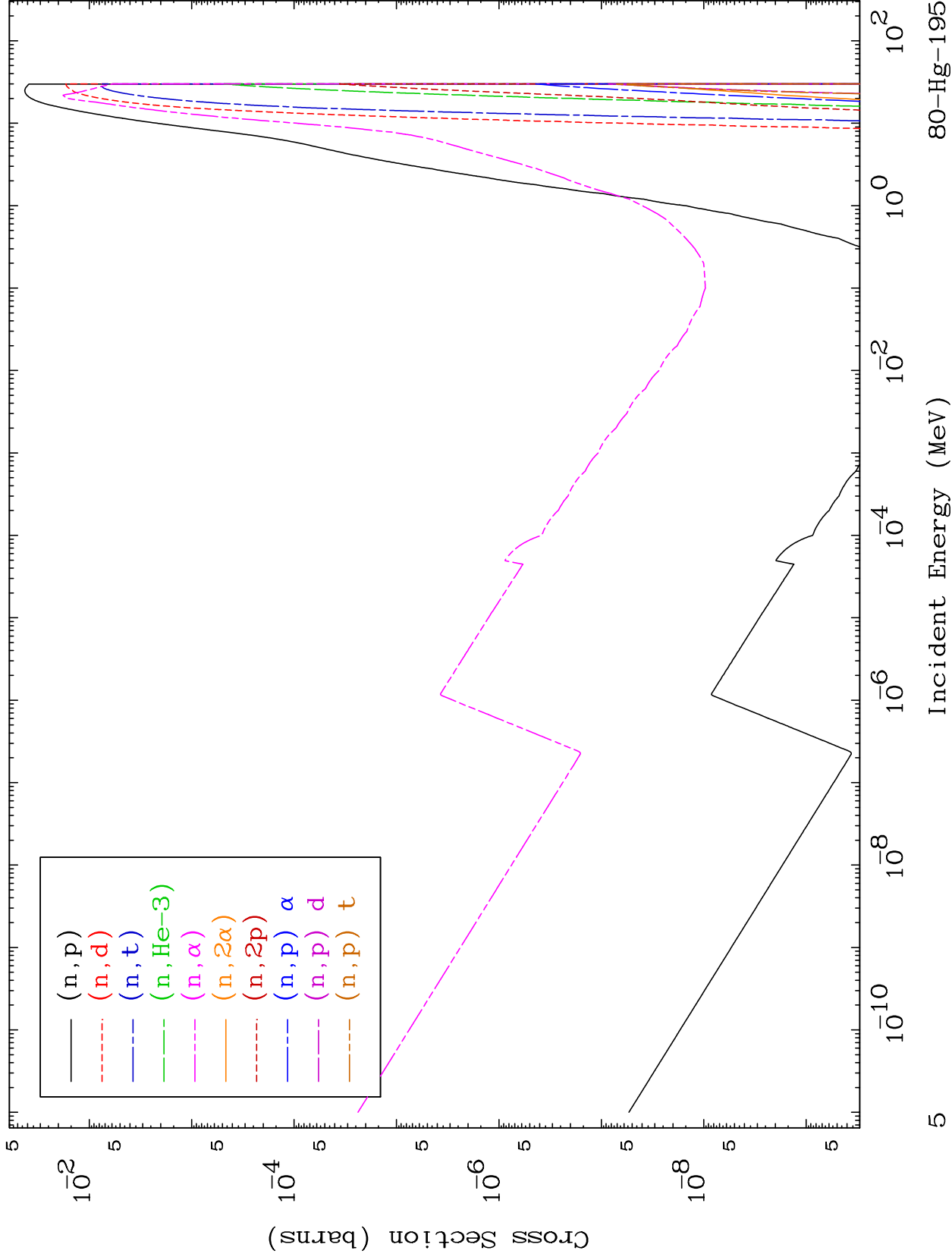
80-Hg-195



MAT 8023

Charged Particle  
293 Kelvin Cross Sections

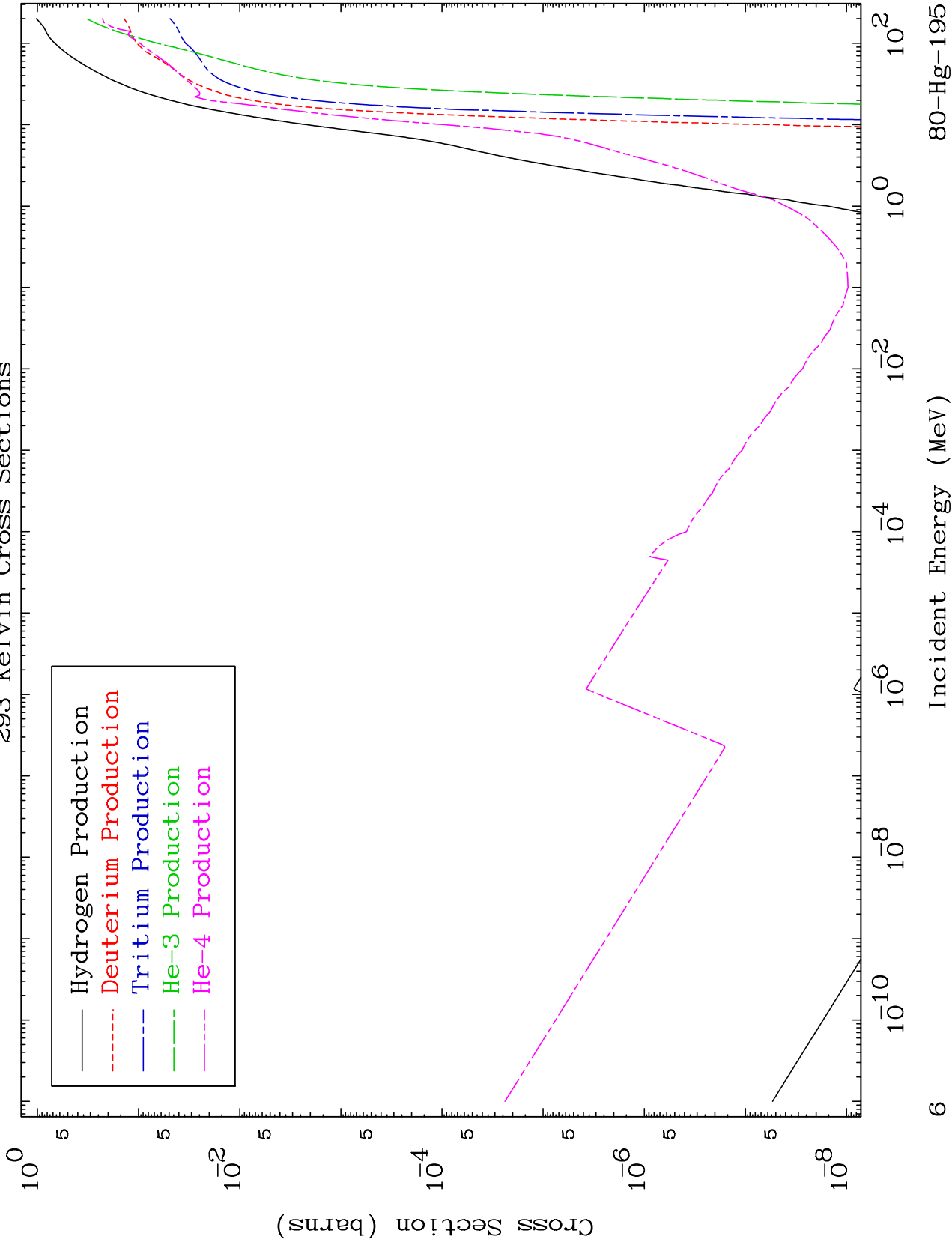
80-Hg-195



MAT 8023

Particle Production  
293 Kelvin Cross Sections

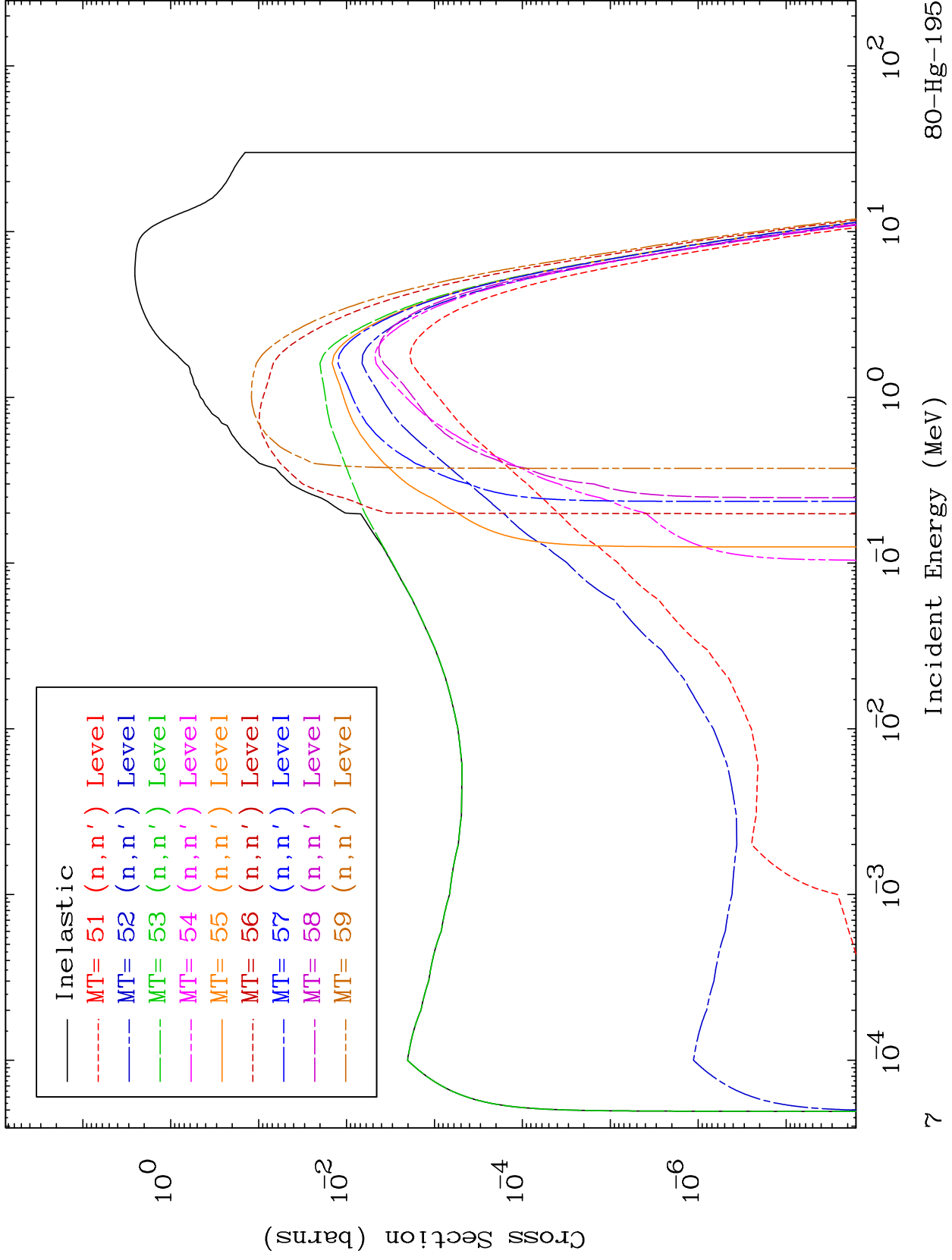
80-Hg-195



MAT 8023

(n,n') Level  
293 Kelvin Cross Sections

80-Hg-195



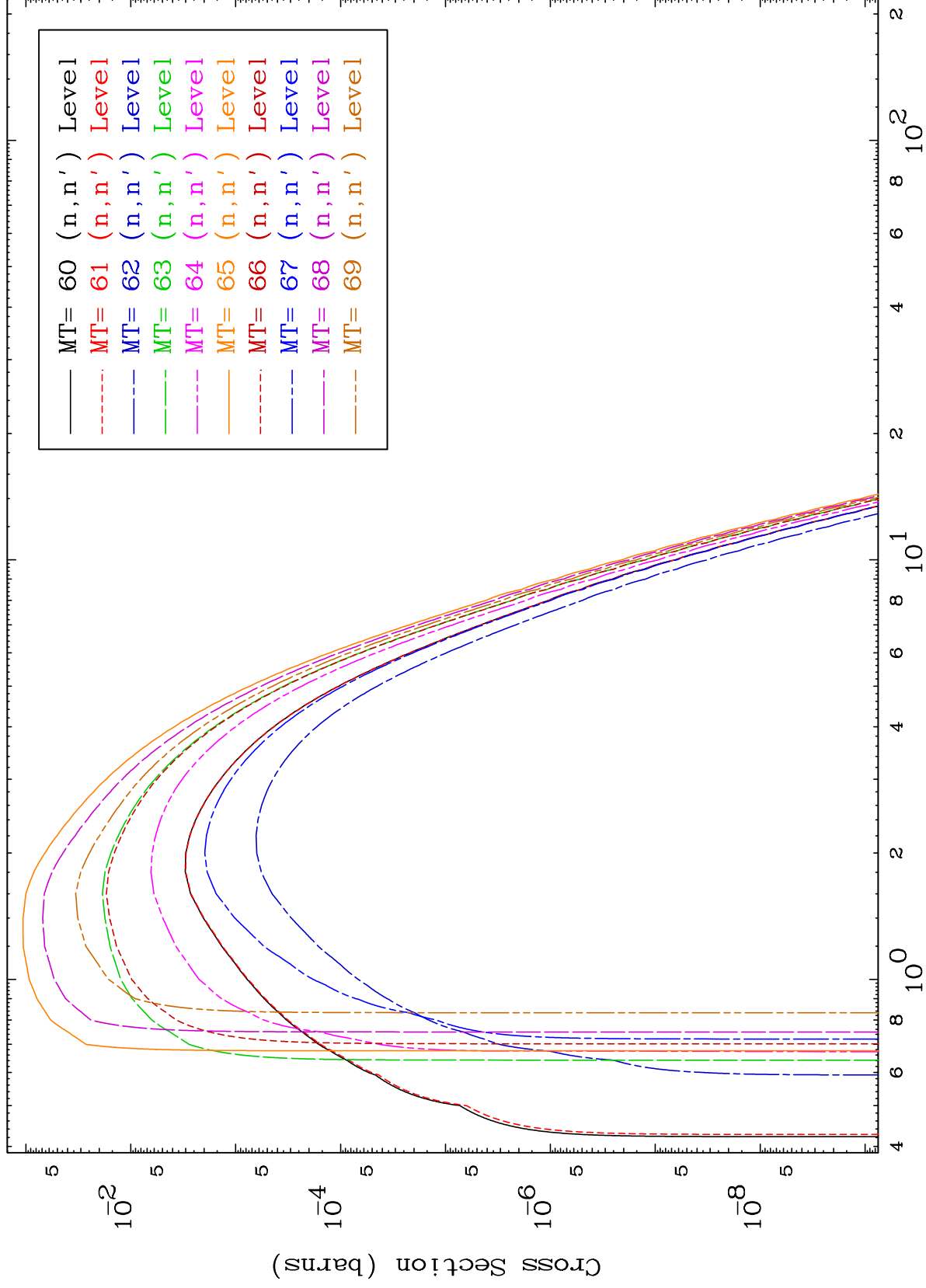


MAT 8023

(n,n') Level

80-Hg-195

293 Kelvin Cross Sections



Incident Energy (MeV)

80-Hg-195

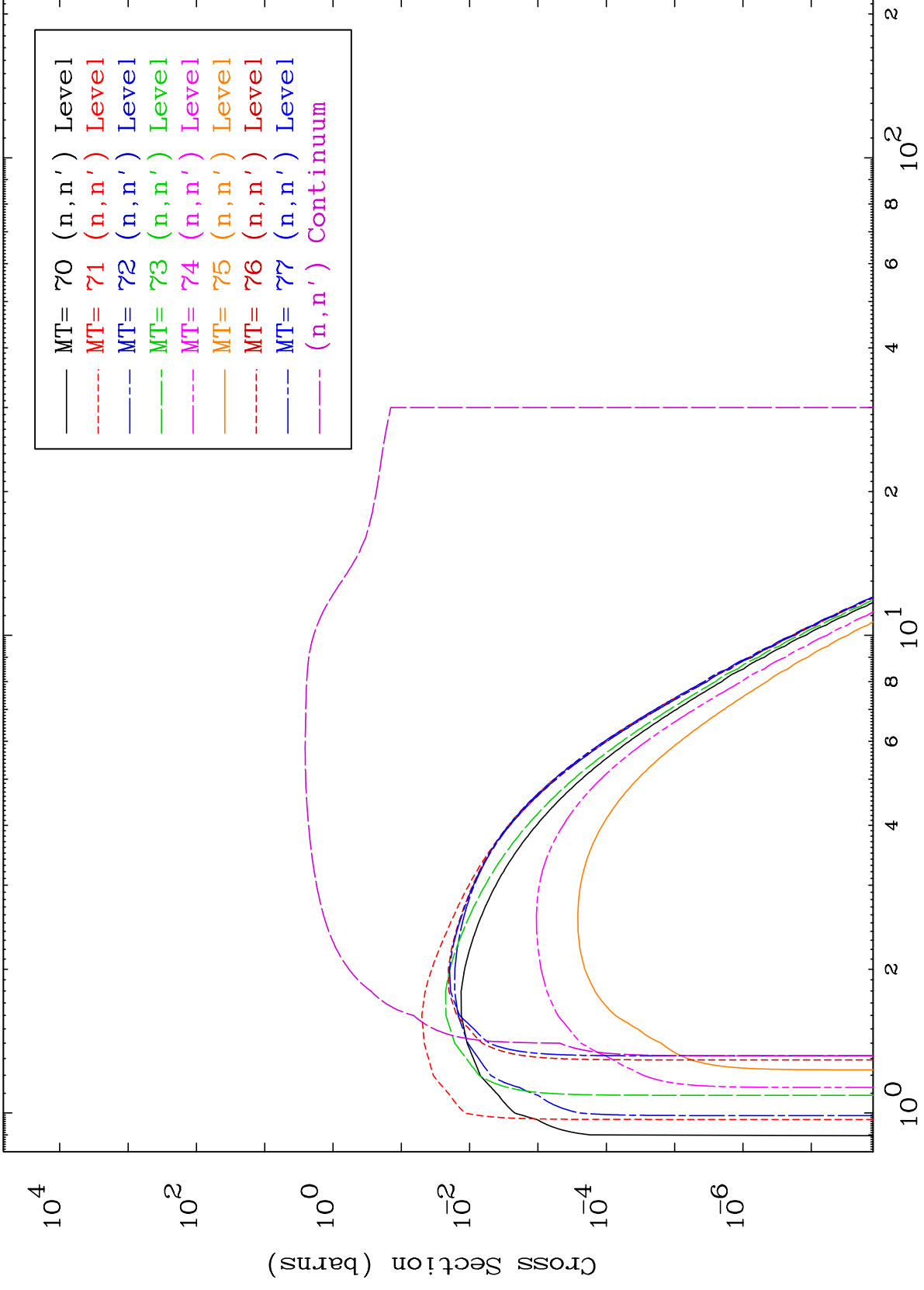
8

MAT 8023

(n,n') Level

80-Hg-195

293 Kelvin Cross Sections



9

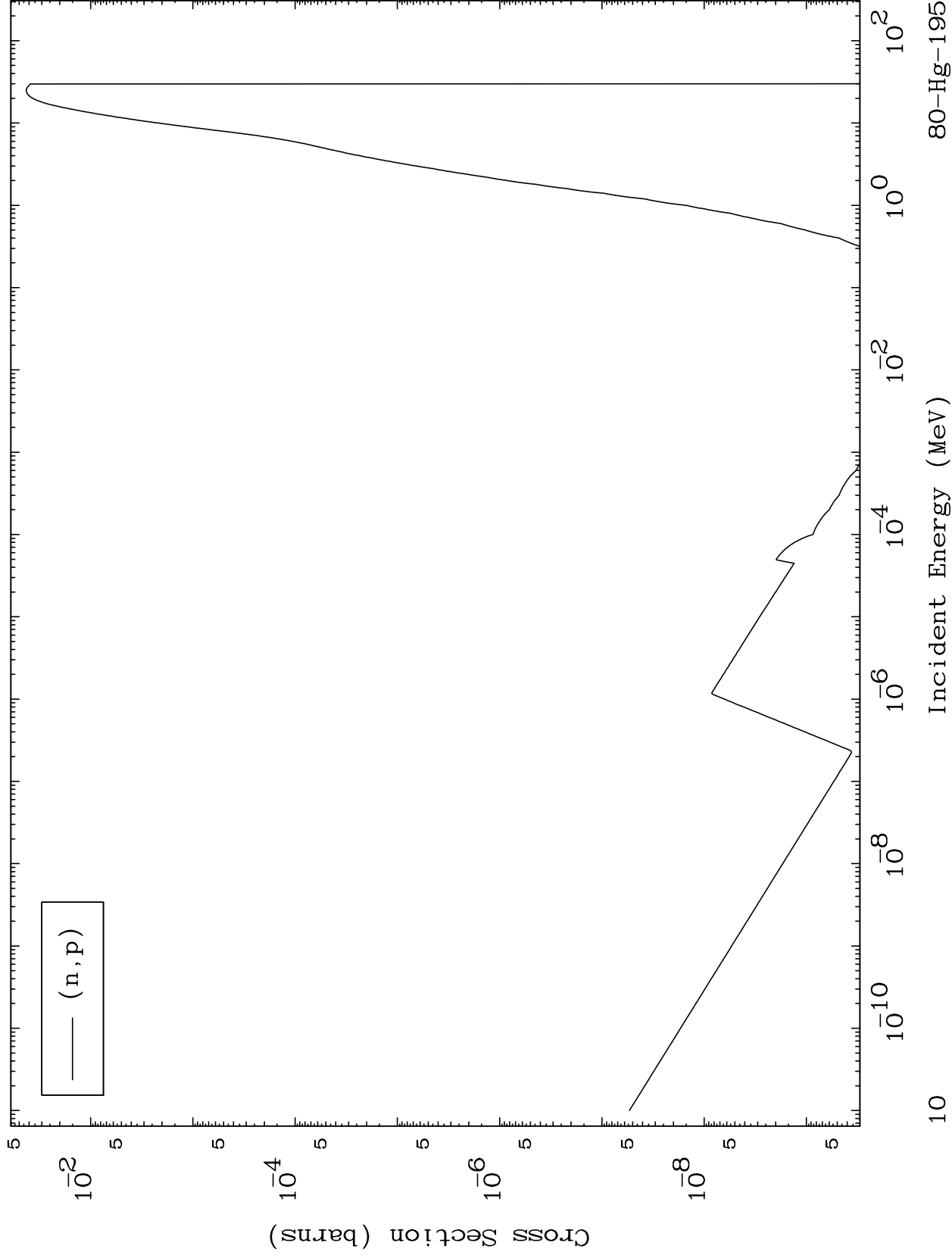
Incident Energy (MeV)

80-Hg-195

MAT 8023

(n,p) Levels  
293 Kelvin Cross Sections

80-Hg-195



10

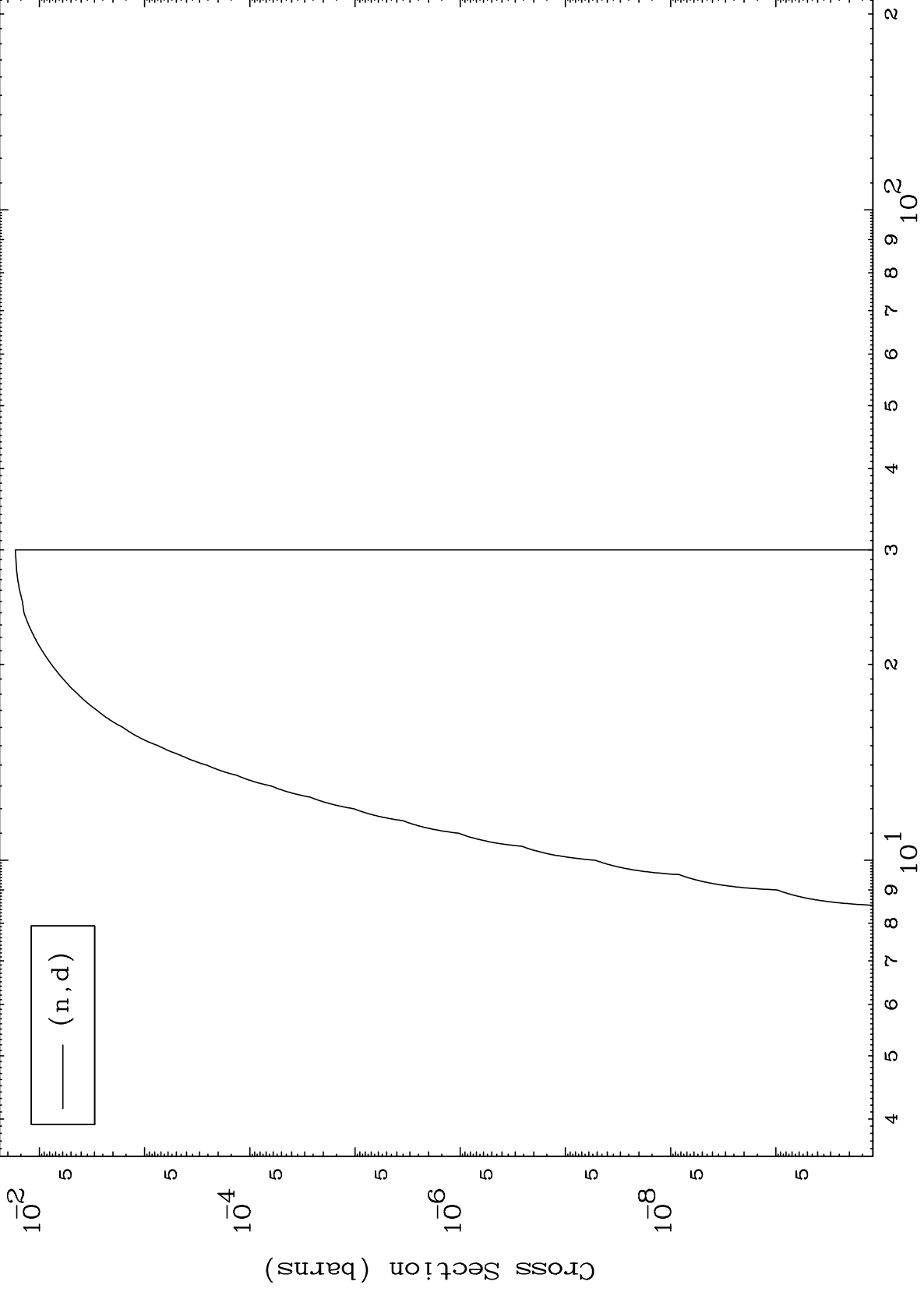
Incident Energy (MeV)

80-Hg-195

MAT 8023

(n,d) Levels  
293 Kelvin Cross Sections

80-Hg-195



11

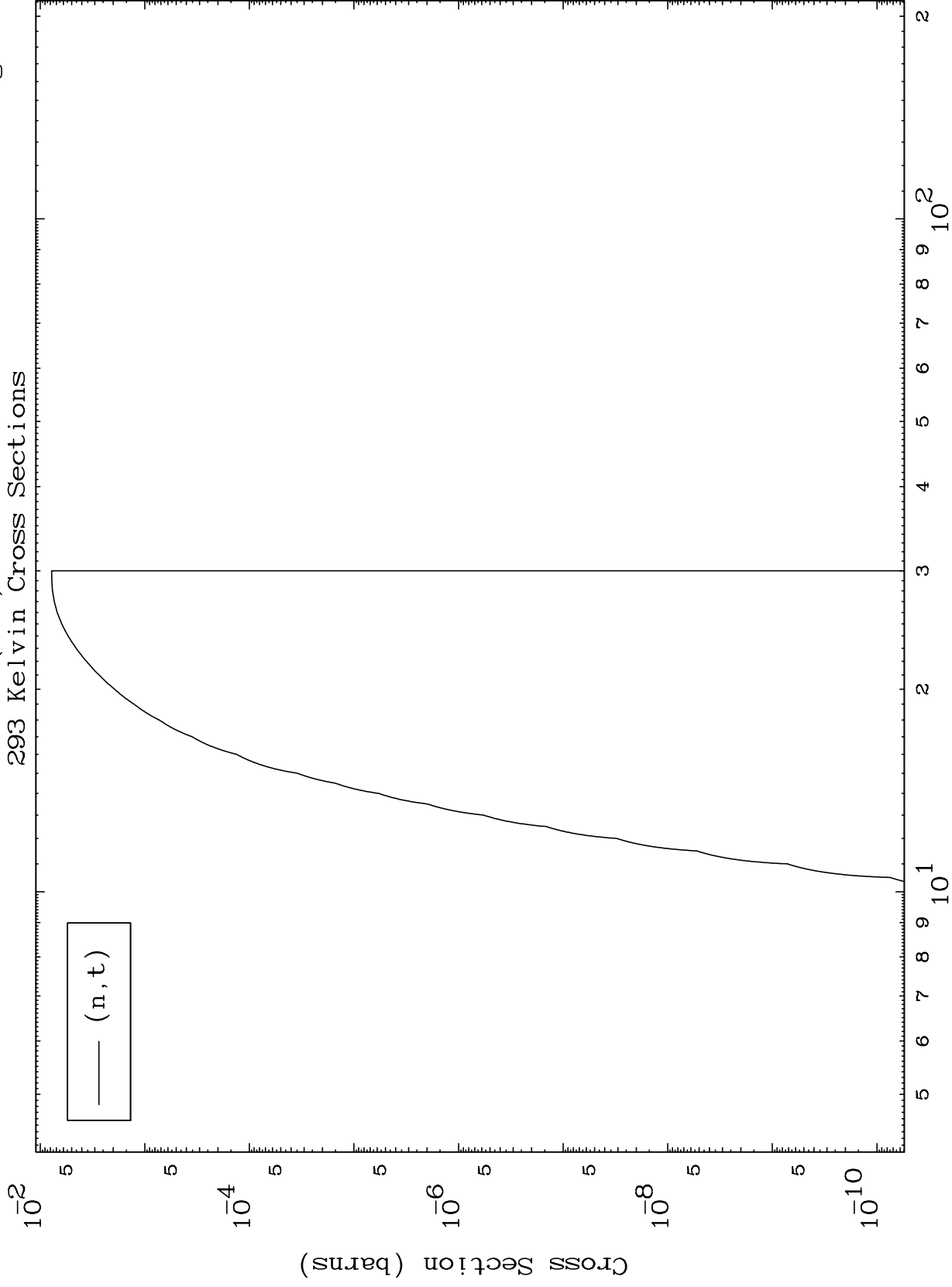
Incident Energy (MeV)

80-Hg-195

MAT 8023

(n,t) Levels  
293 Kelvin Cross Sections

80-Hg-195



12

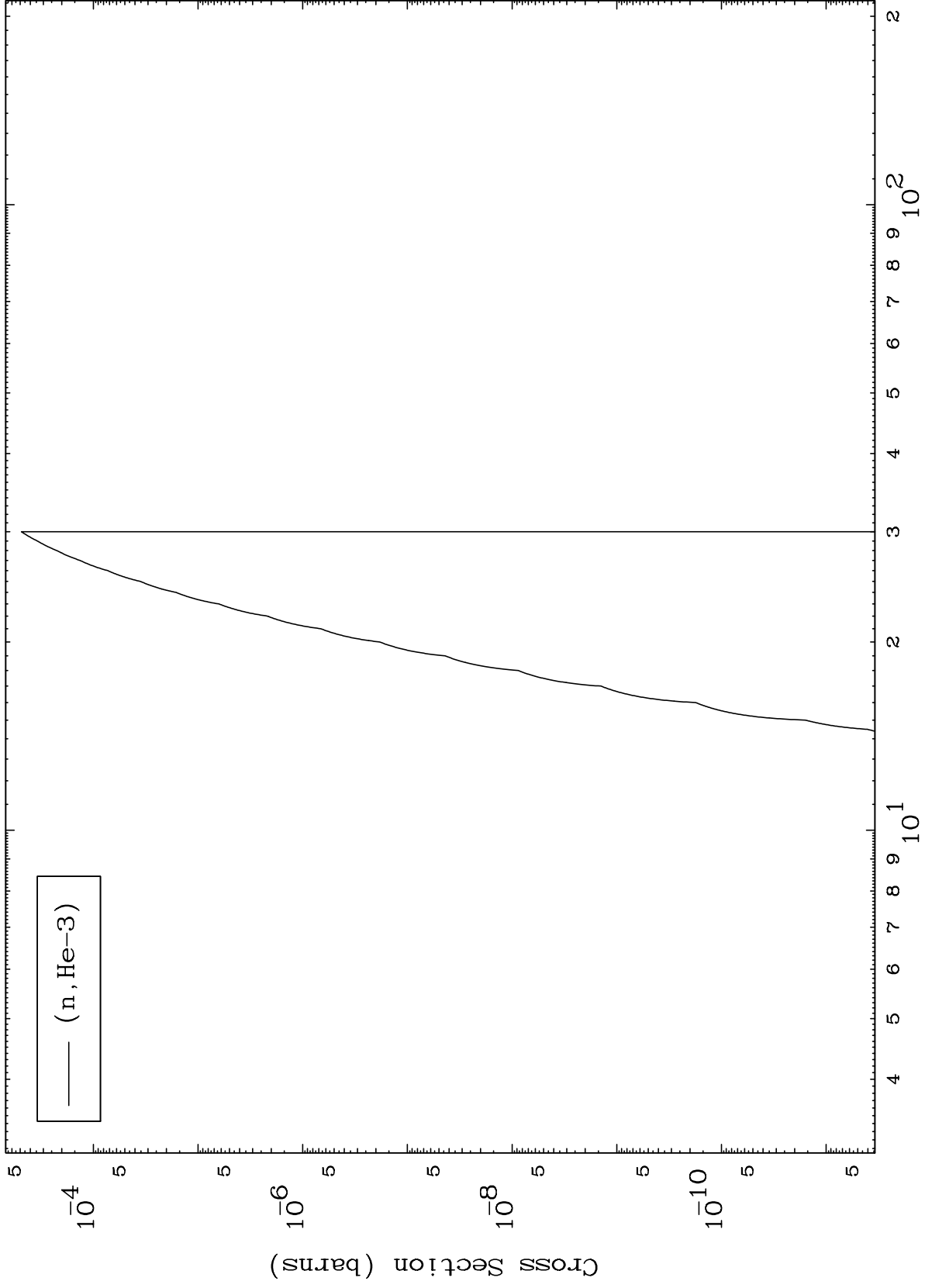
Incident Energy (MeV)

80-Hg-195

MAT 8023

(n,He3) Levels  
293 Kelvin Cross Sections

80-Hg-195



13

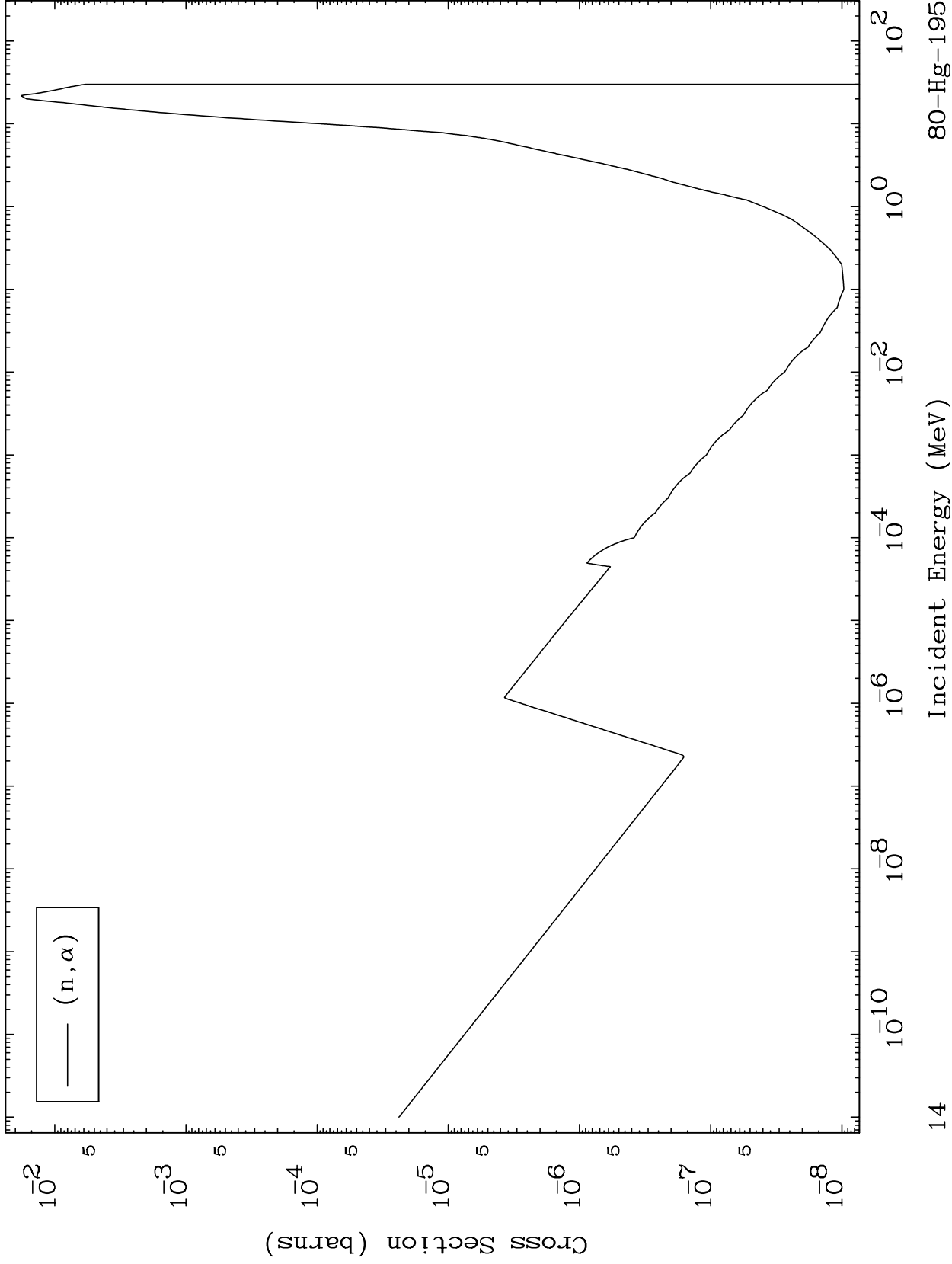
Incident Energy (MeV)

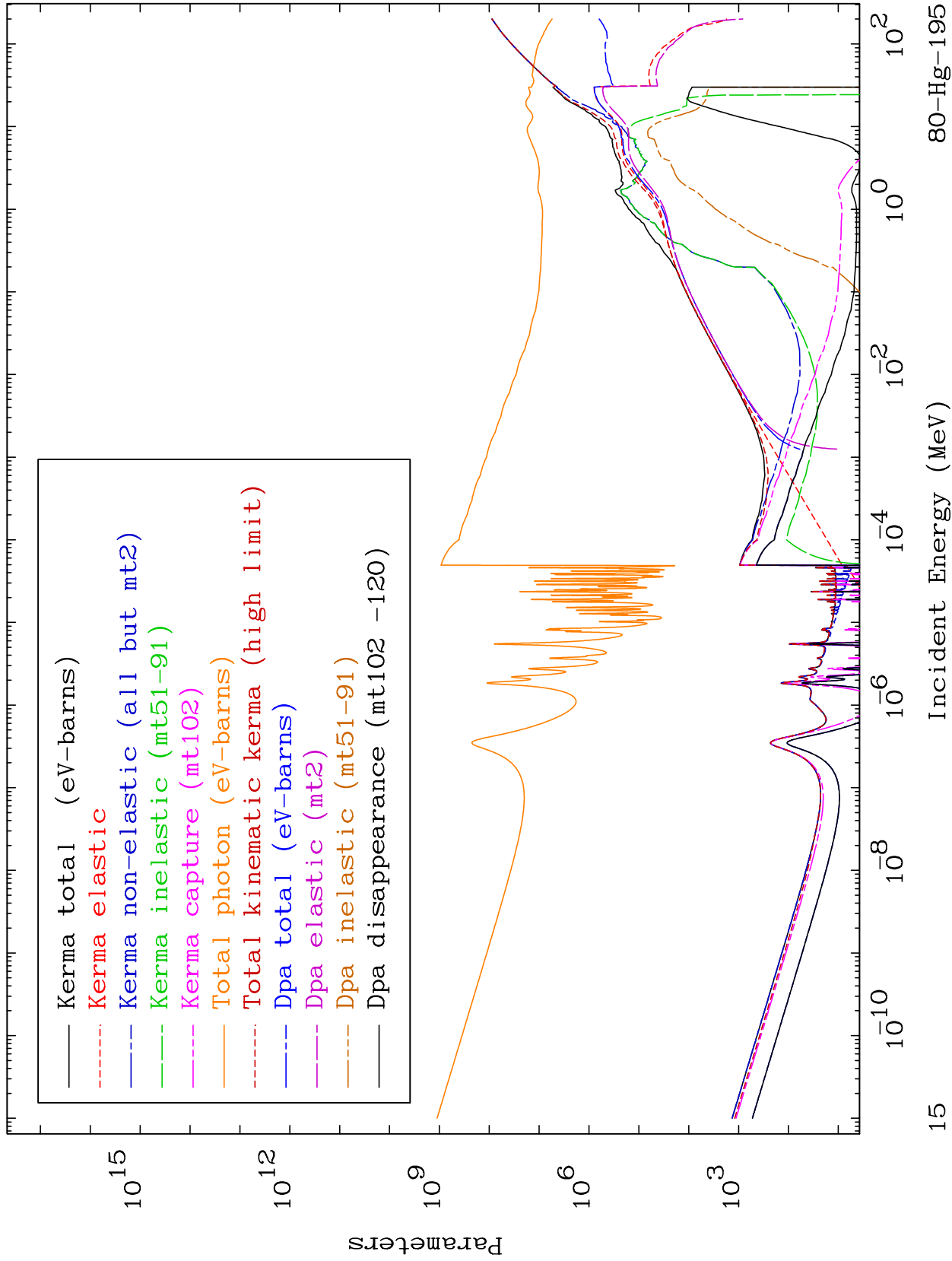
80-Hg-195

MAT 8023

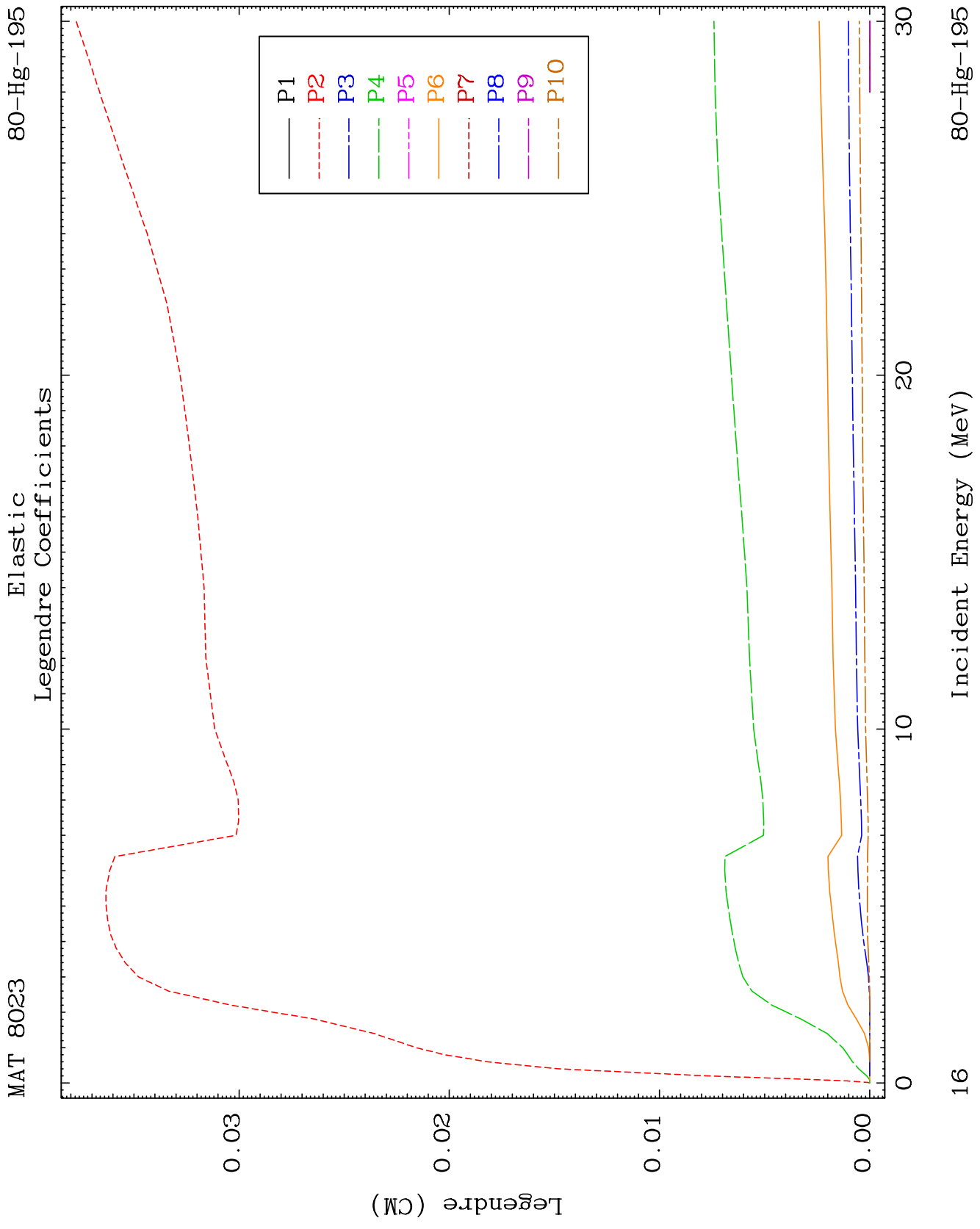
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

80-Hg-195







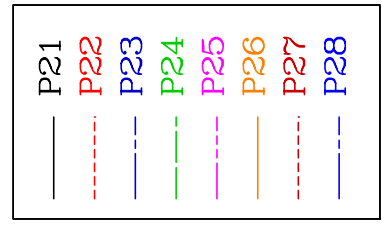




MAT 8023

Elastic Legendre Coefficients

80-Hg-195



$\times 10^{-6}$   
1.5  
1.0  
0.5  
0.0

Legendre (CM)



15 20 25 30

18

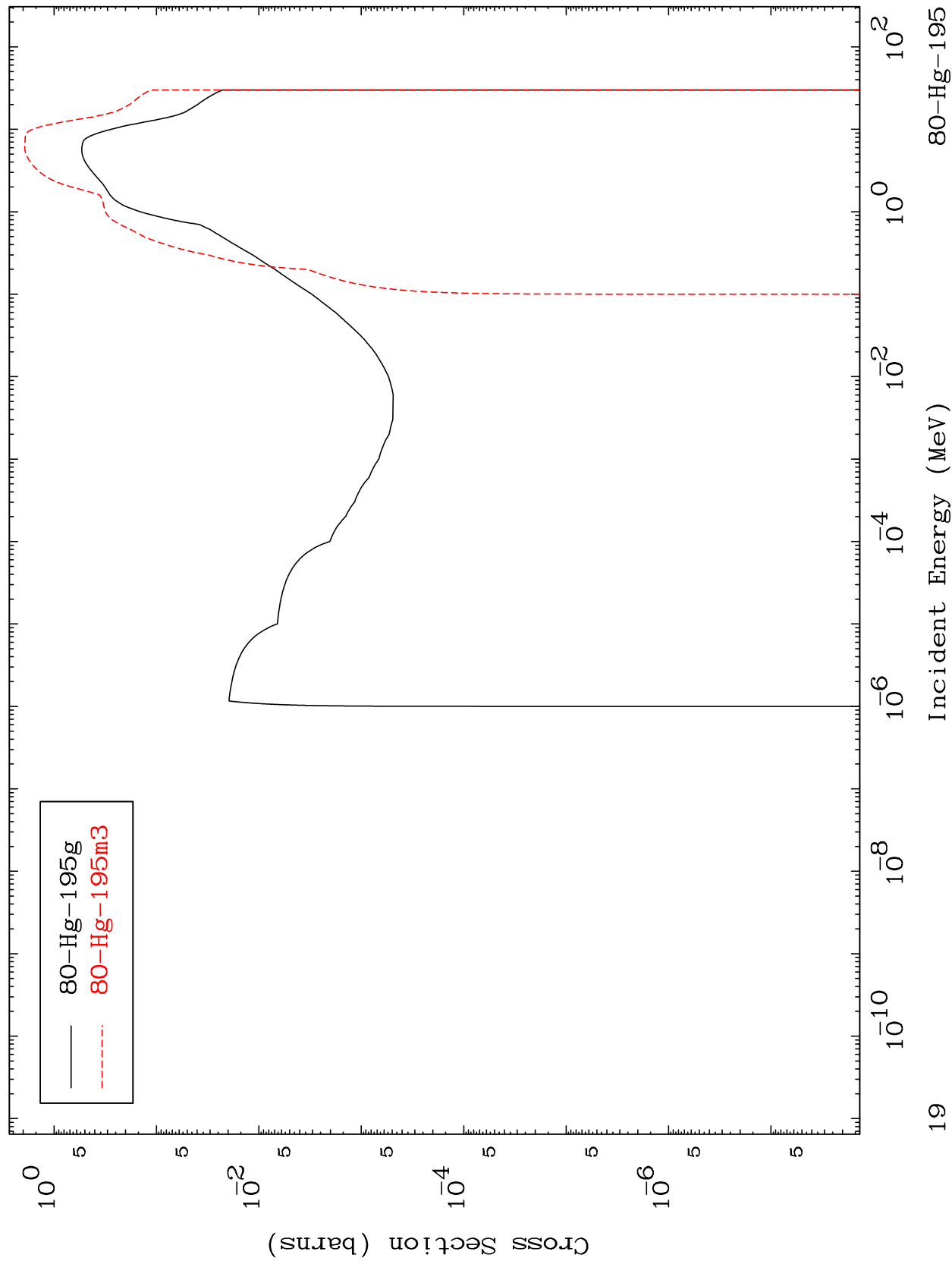
Incident Energy (MeV)

80-Hg-195

MAT 8023

80-Hg-195

Inelastic  
Radionuclide Production Cross Section



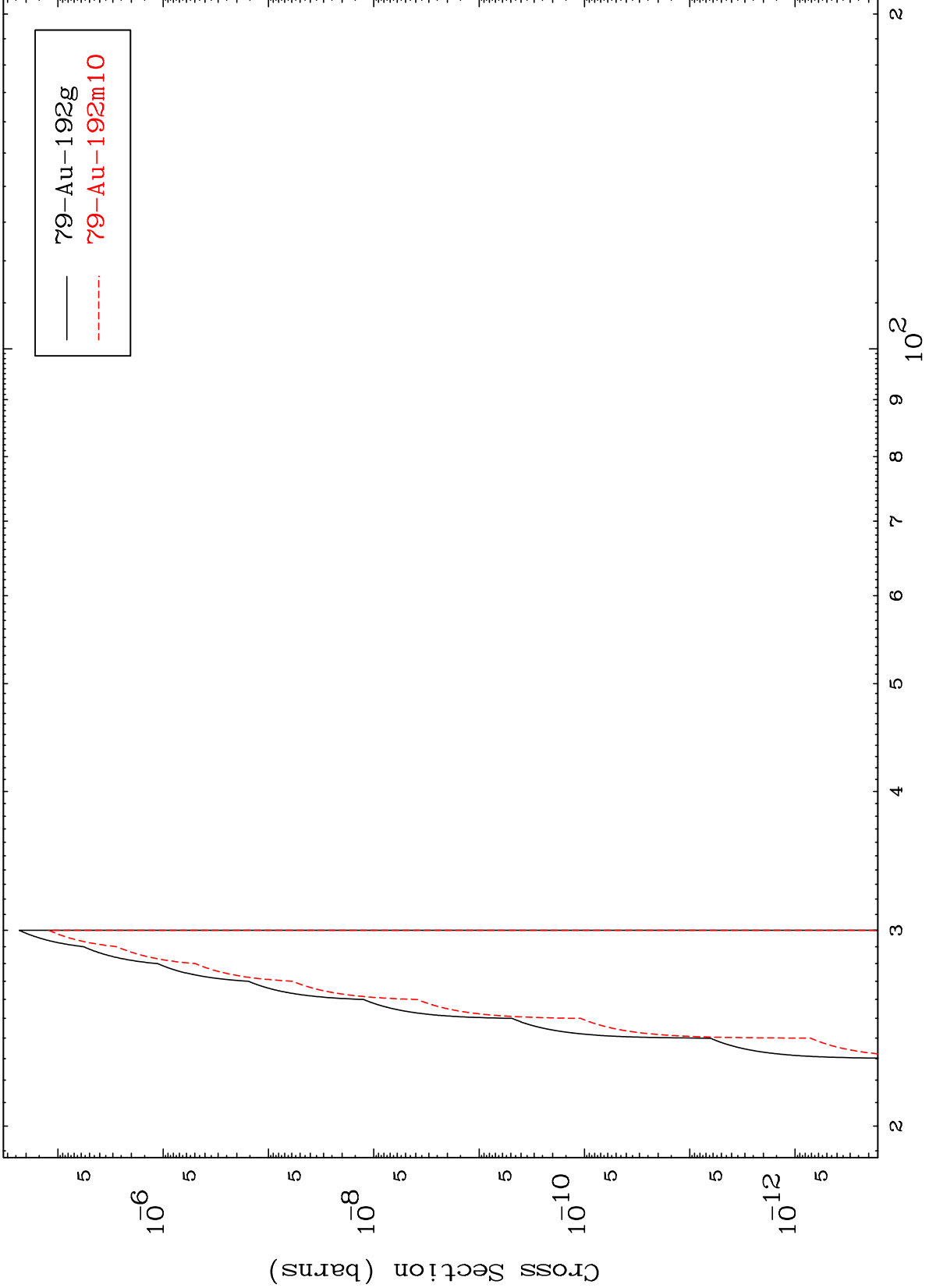
80-Hg-195g  
80-Hg-195m3

MAT 8023

(n,2n) d

80-Hg-195

Radionuclide Production Cross Section



20

Incident Energy (MeV)

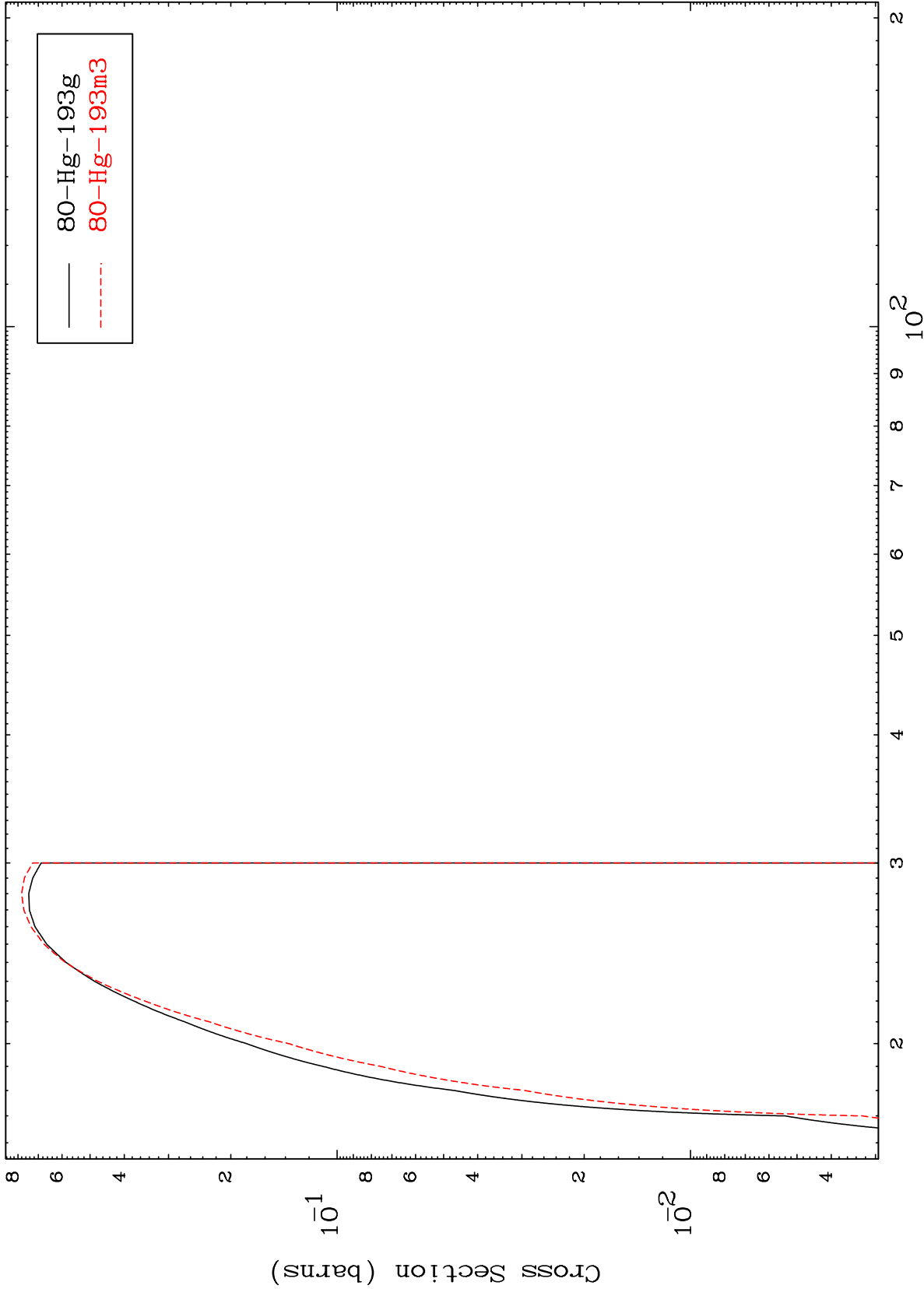
80-Hg-195

MAT 8023

(n,3n)

80-Hg-195

Radionuclide Production Cross Section



80-Hg-193g  
80-Hg-193m3

21

Incident Energy (MeV)

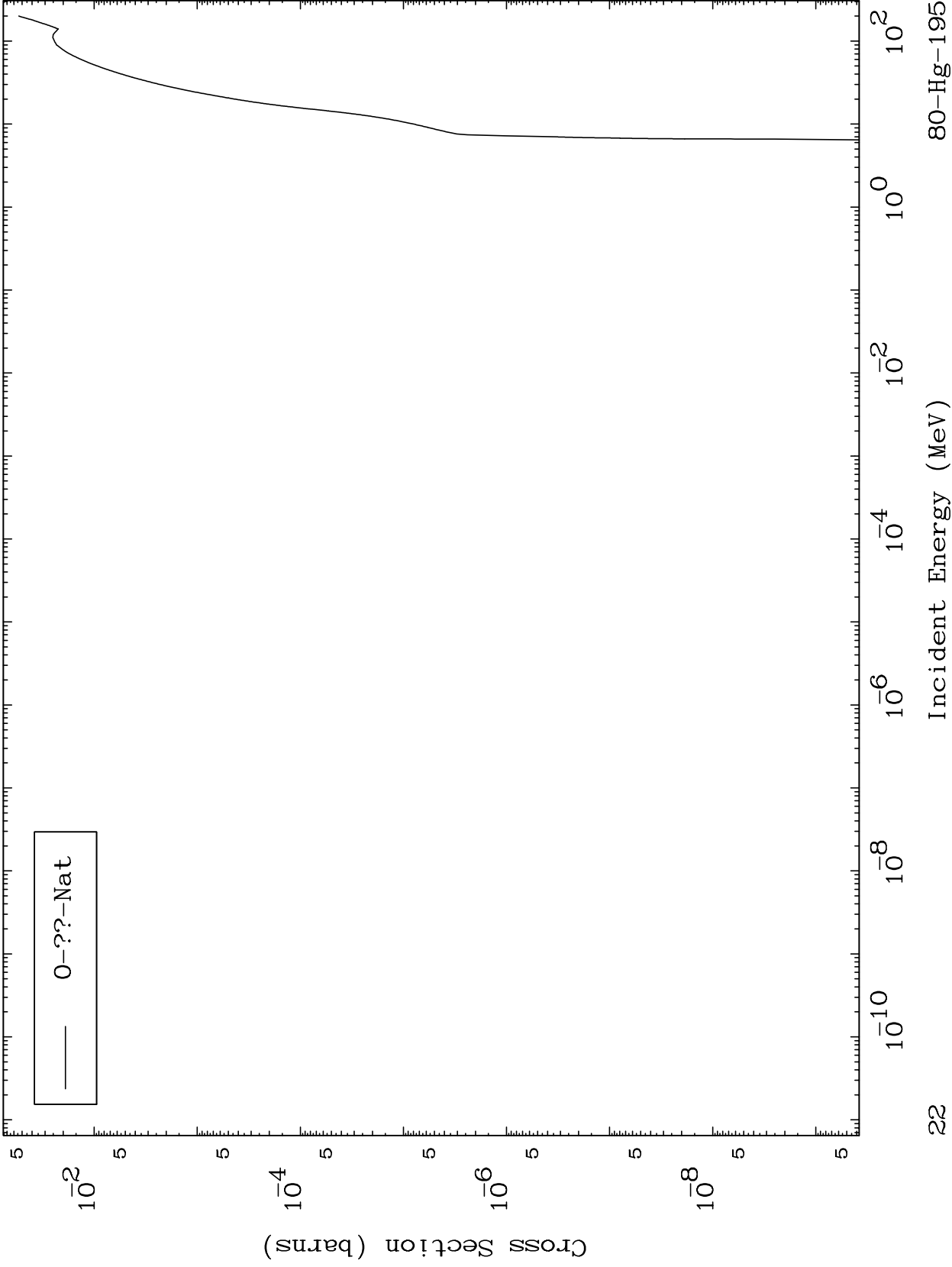
80-Hg-195

MAT 8023

Fission

80-Hg-195

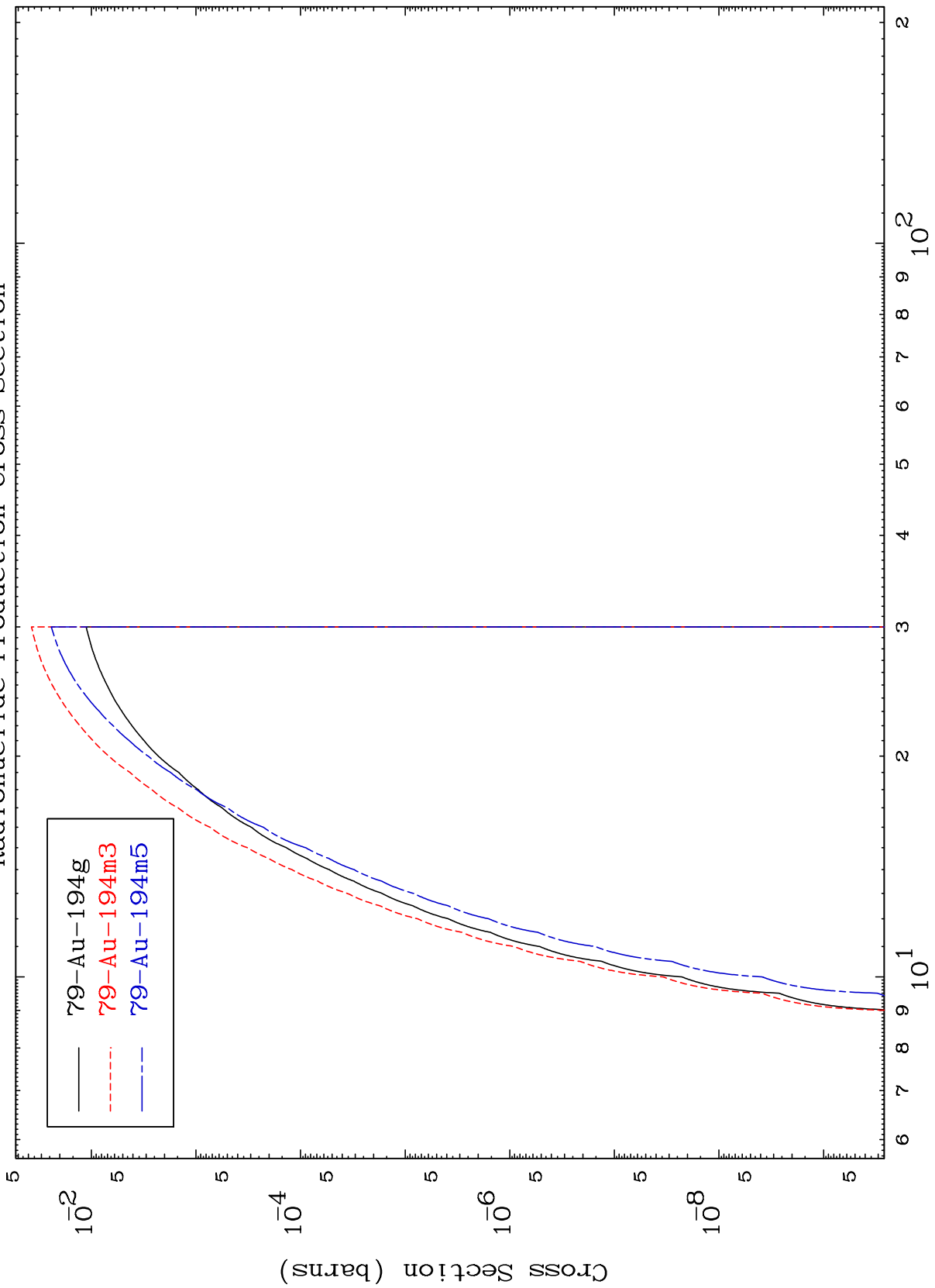
Radionuclide Production Cross Section



MAT 8023

80-Hg-195

(n,n') p  
Radionuclide Production Cross Section



23

Incident Energy (MeV)

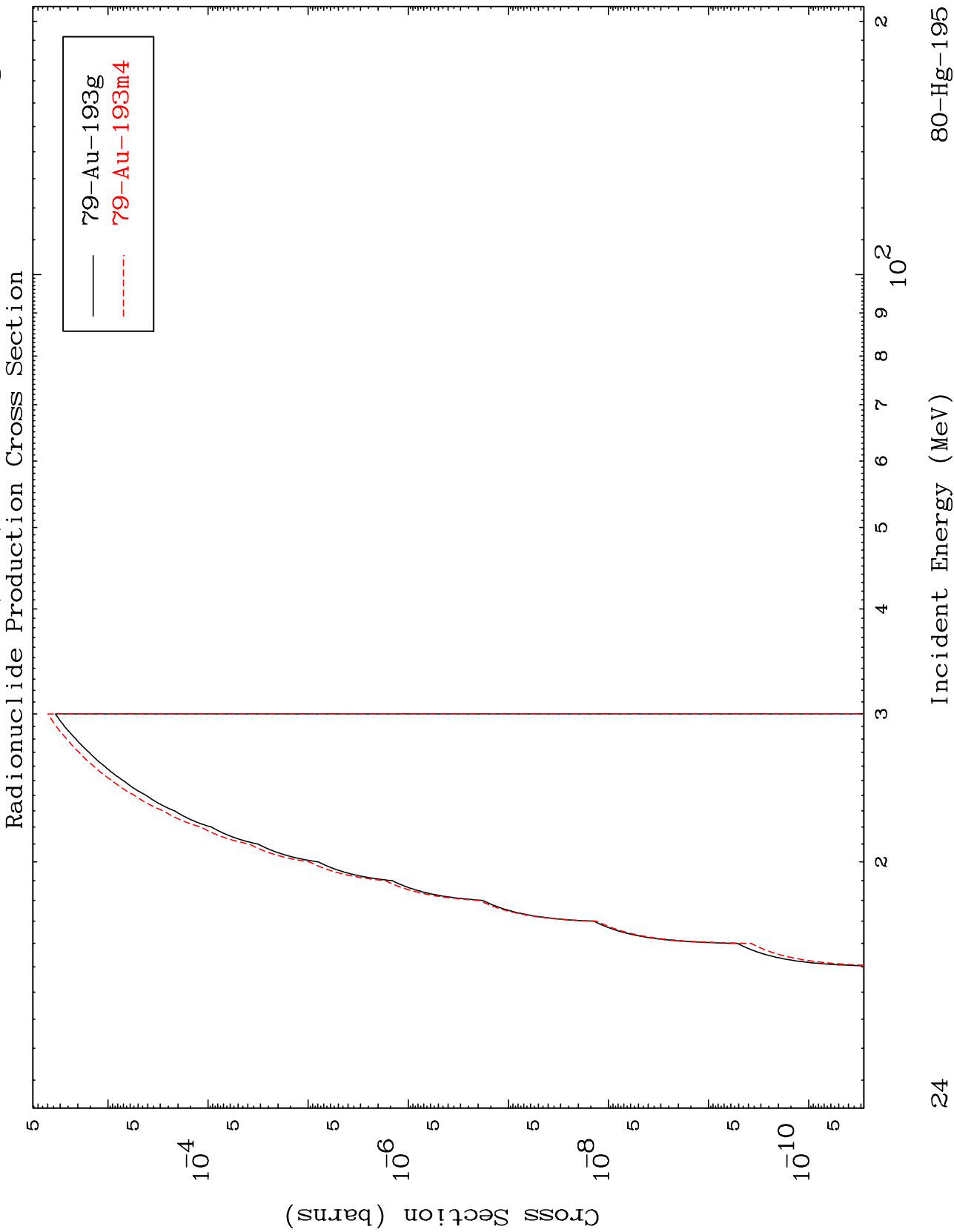
80-Hg-195



MAT 8023

(n,n') d

80-Hg-195

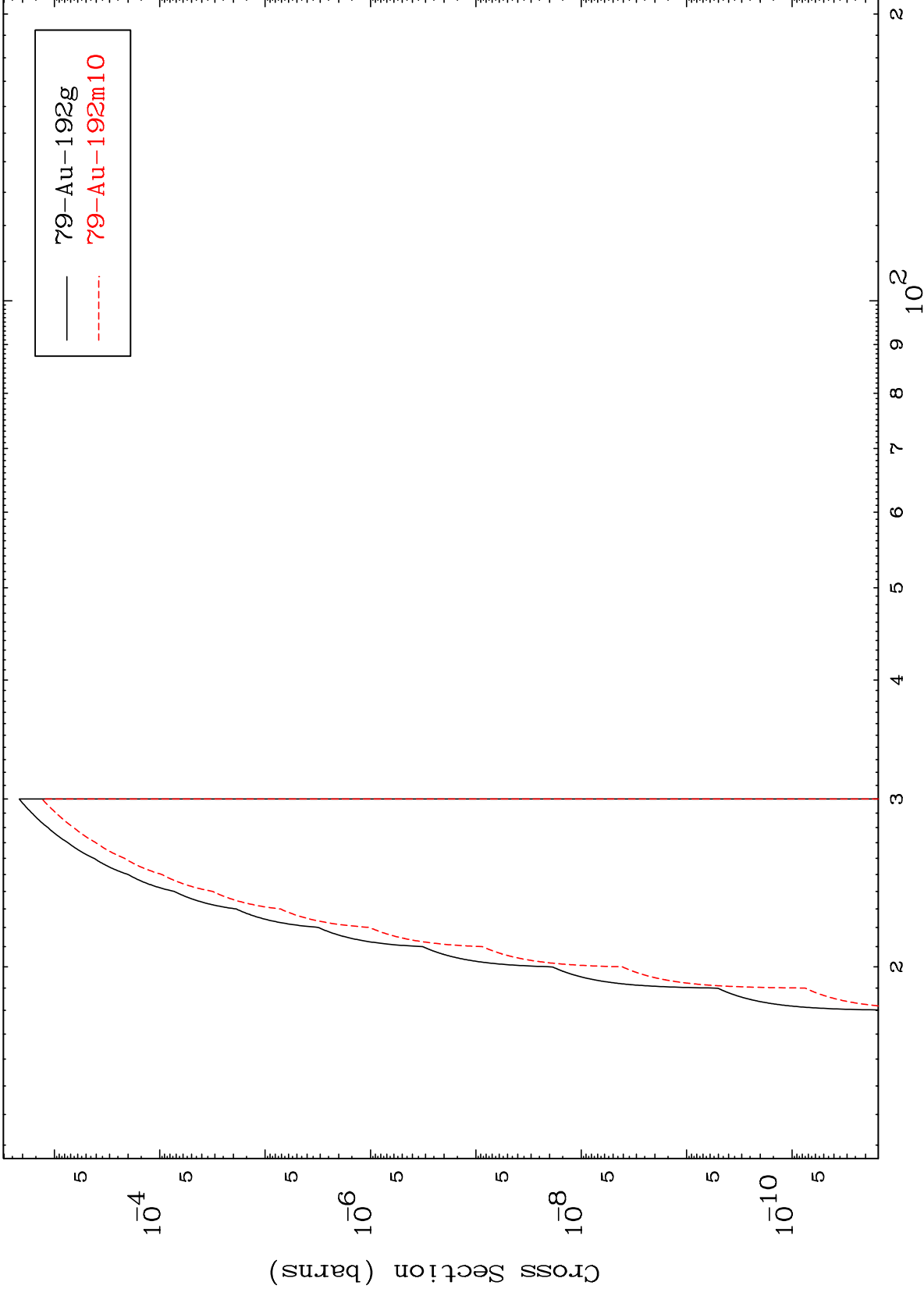


24

80-Hg-195

80-Hg-195

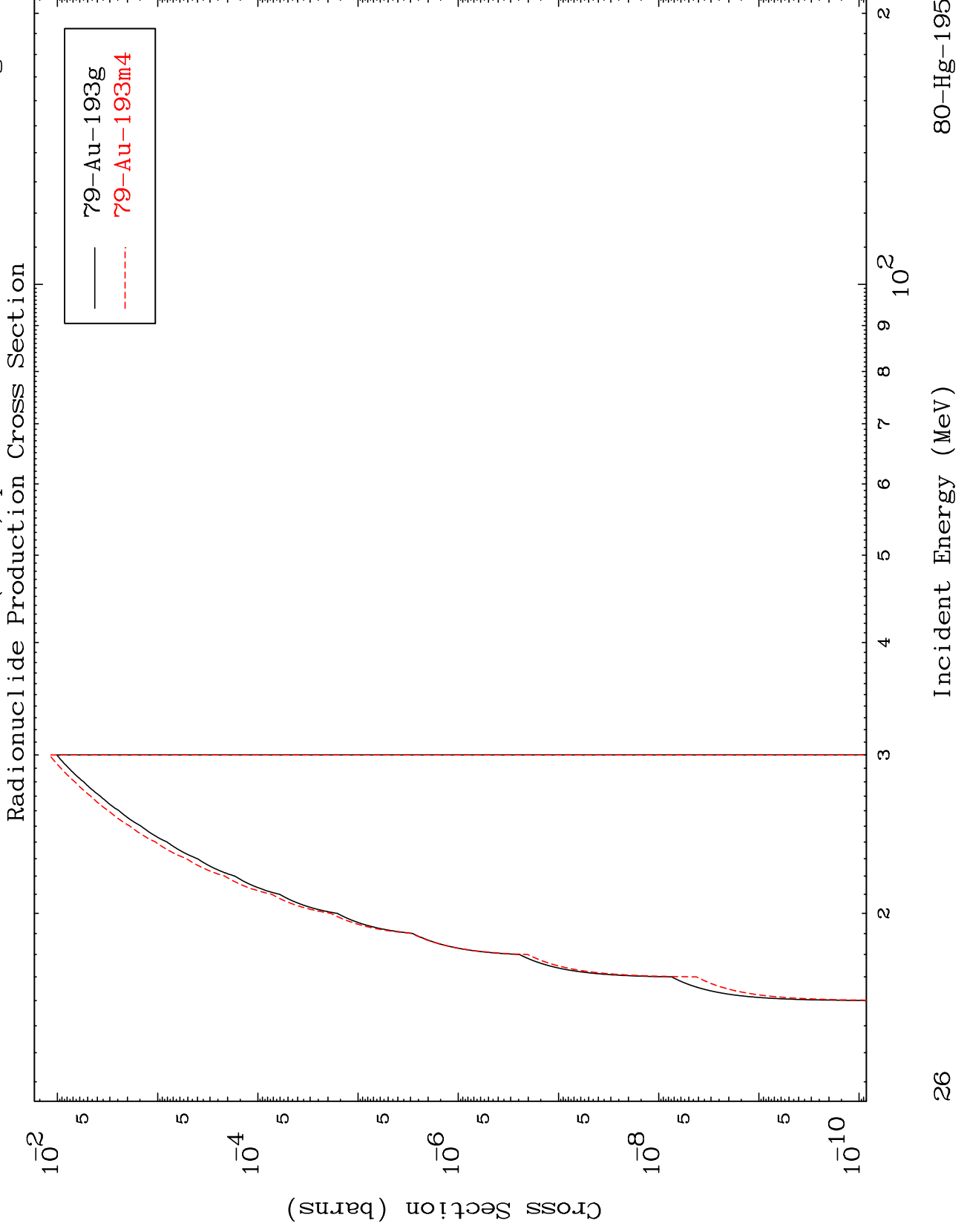
Radionuclide Production Cross Section



MAT 8023

(n,2n) p

80-Hg-195



26

Incident Energy (MeV)

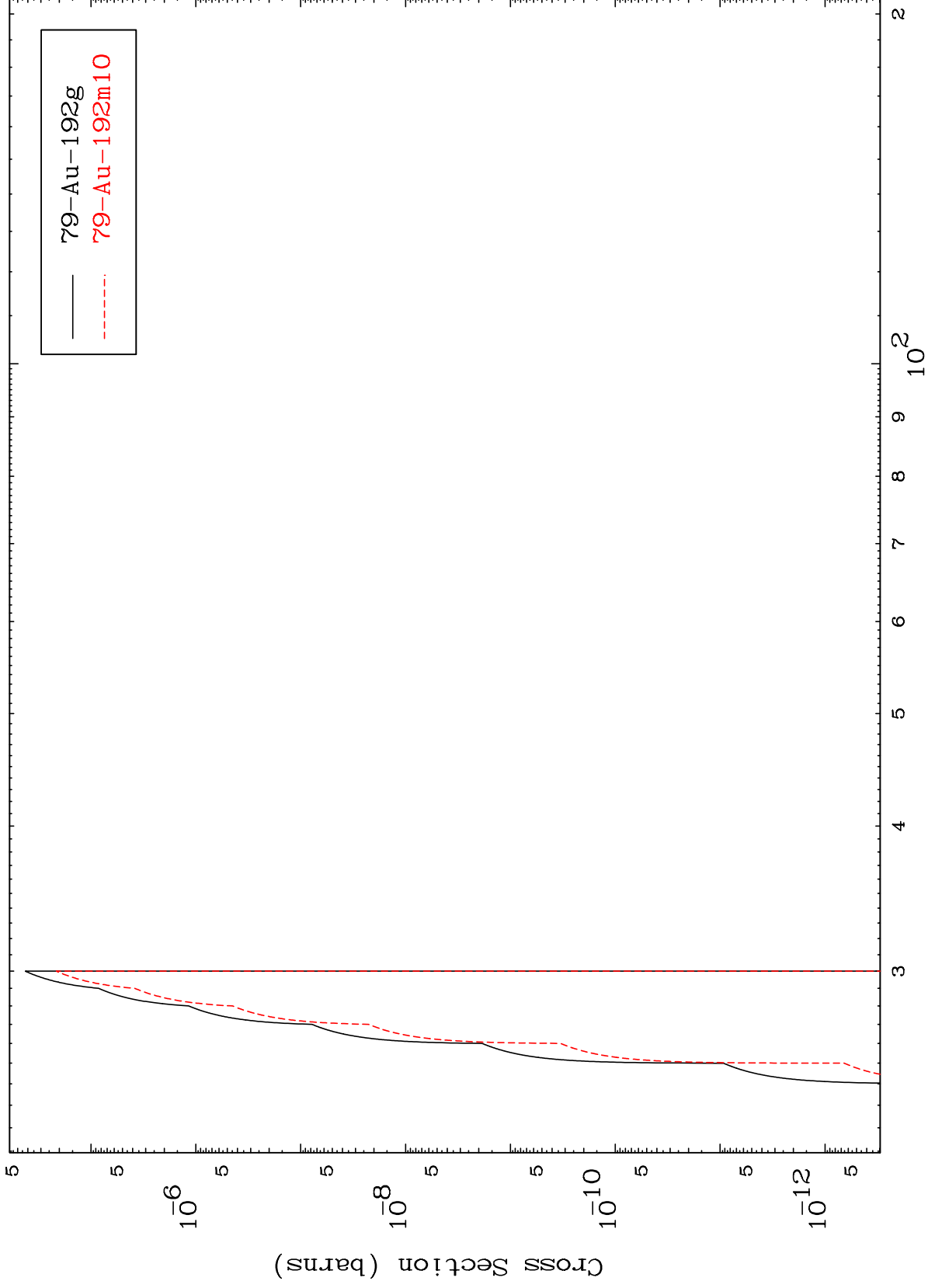
80-Hg-195

MAT 8023

(n,3n) p

80-Hg-195

Radionuclide Production Cross Section

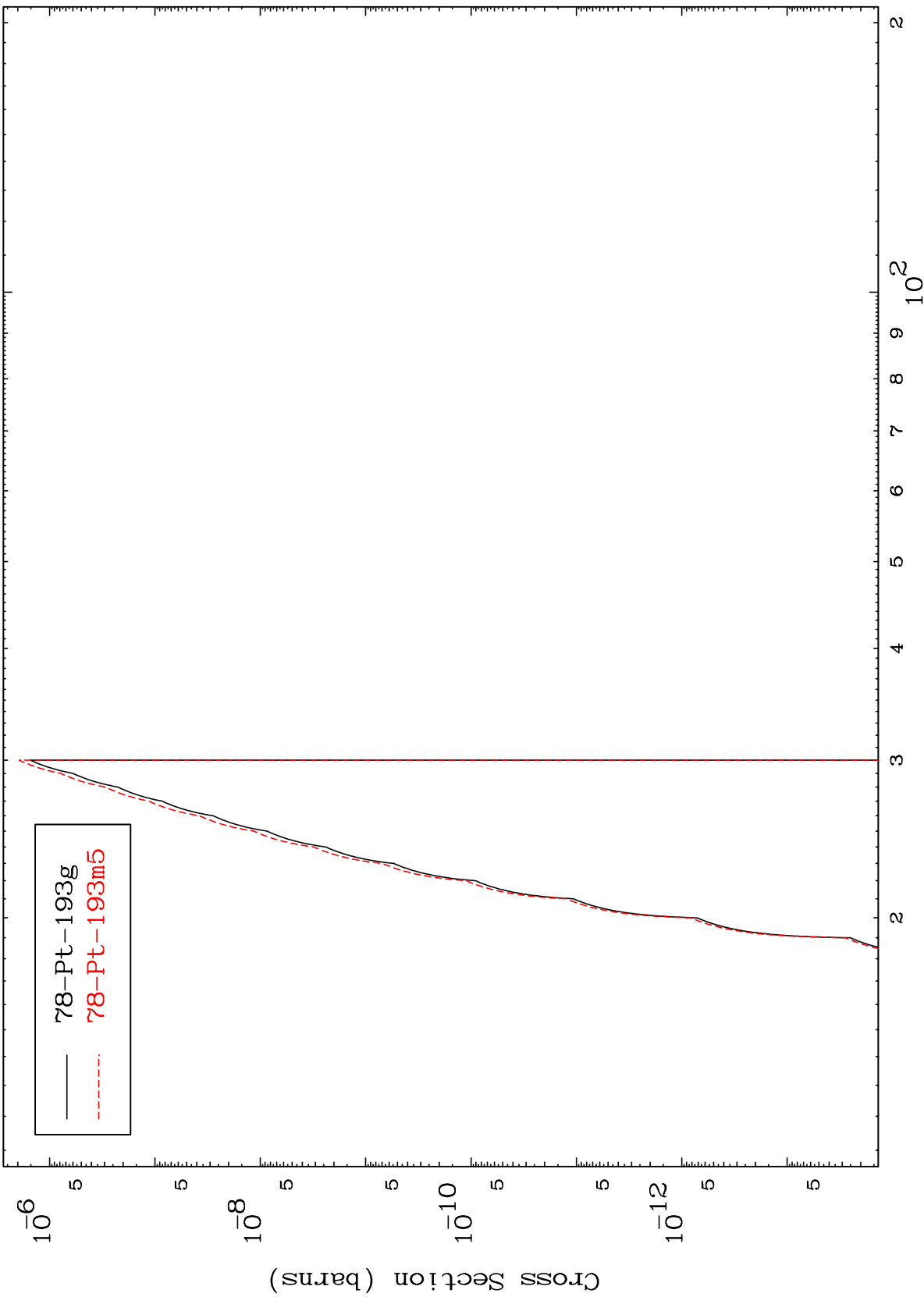


27

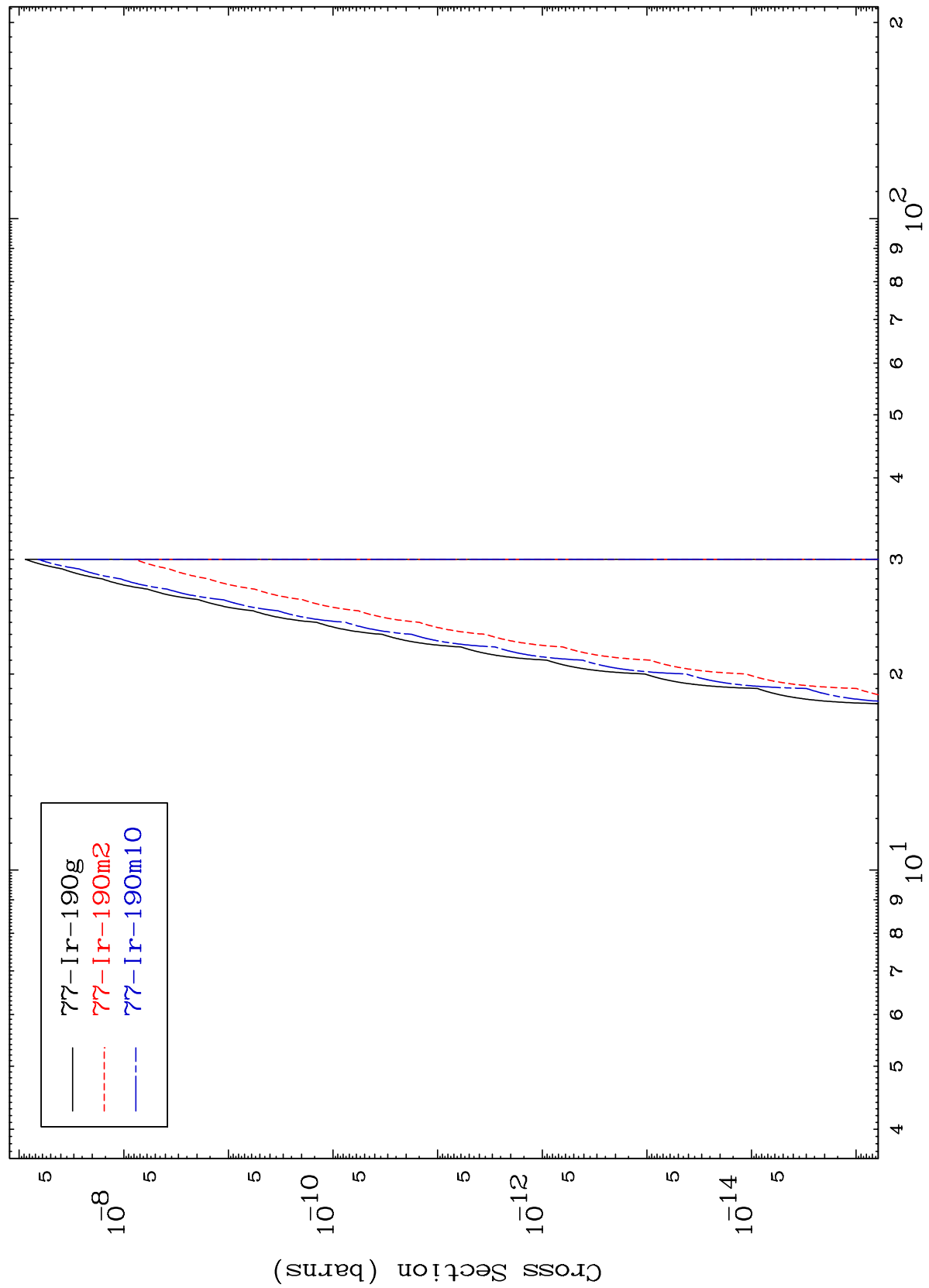
Incident Energy (MeV)

80-Hg-195

Radionuclide Production Cross Section



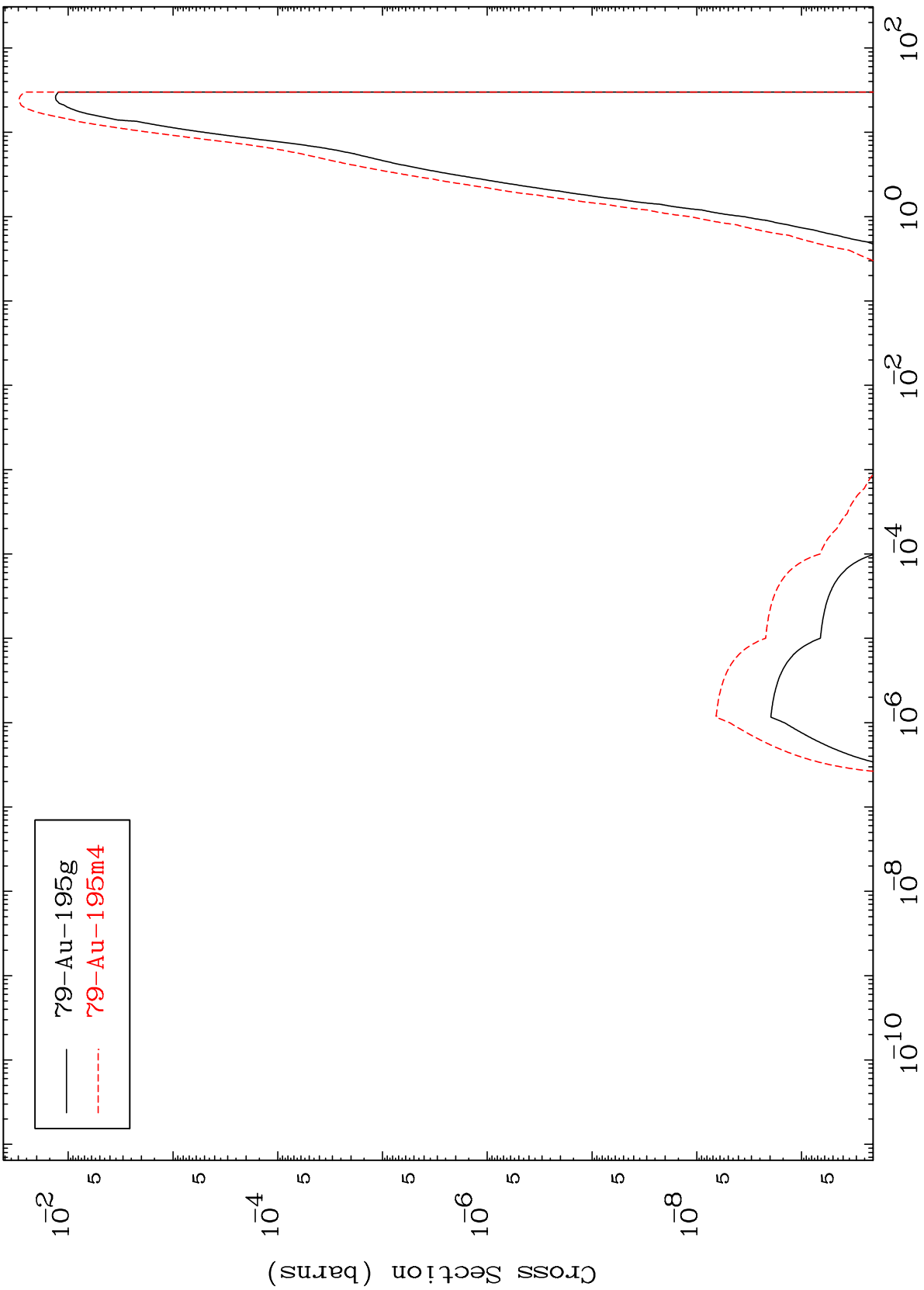
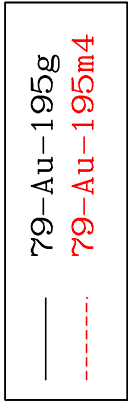
Radionuclide Production Cross Section



MAT 8023

(n,p)  
Radionuclide Production Cross Section

80-Hg-195



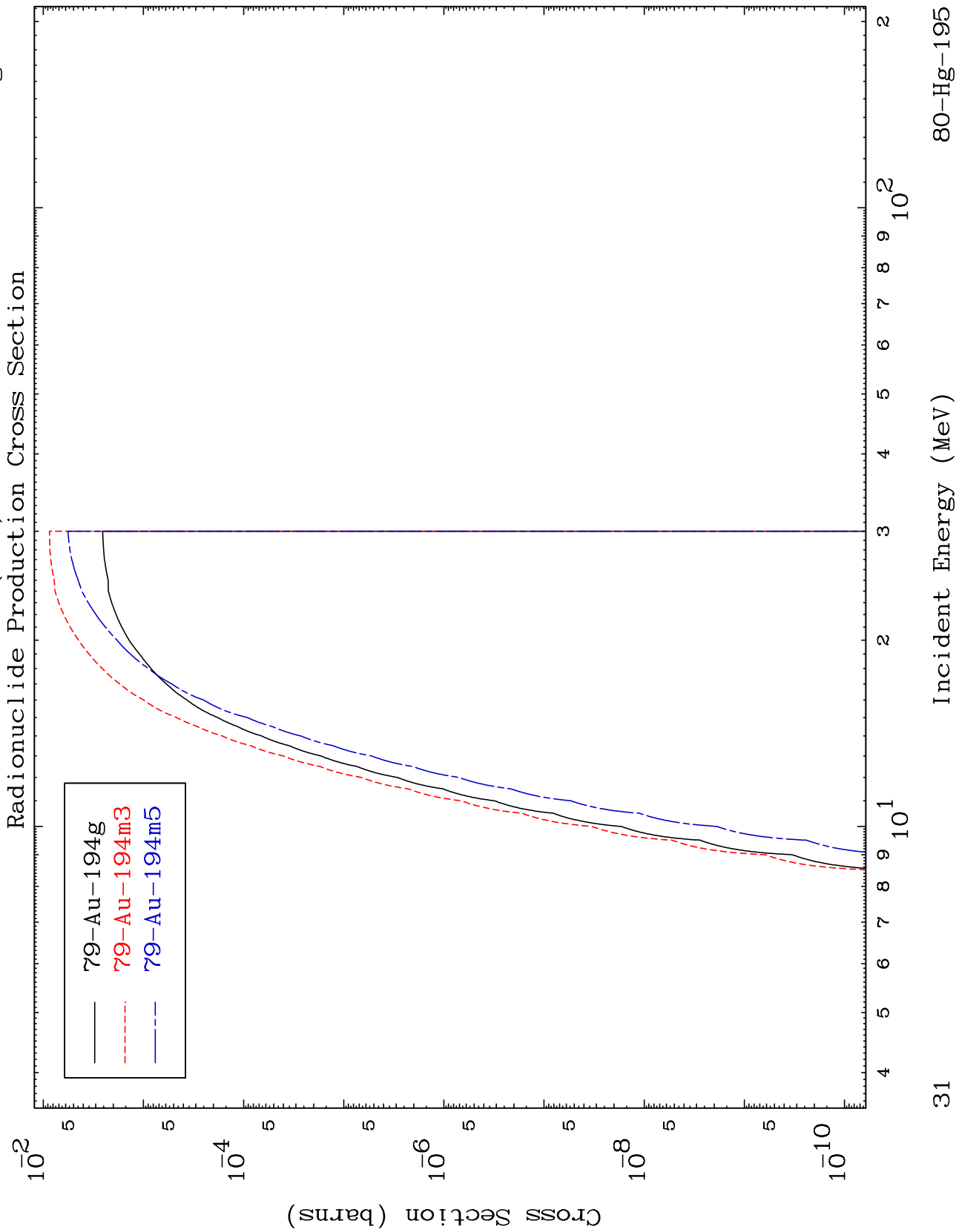
30

Incident Energy (MeV)

80-Hg-195

MAT 8023

80-Hg-195



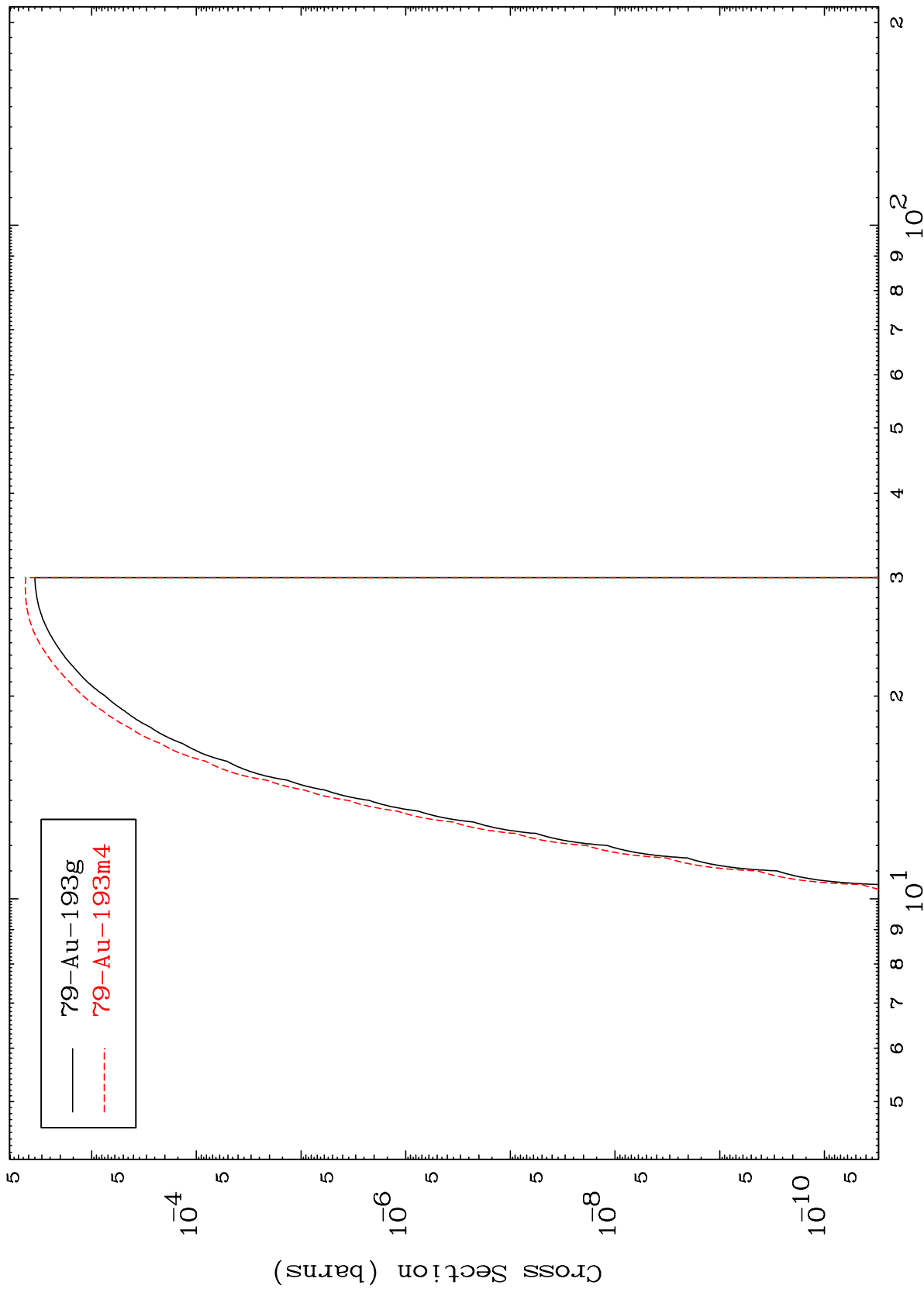
79-Au-194g  
79-Au-194m3  
79-Au-194m5



MAT 8023

80-Hg-195

Radionuclide Production Cross Section (n, t)



79-Au-193g  
79-Au-193m4

80-Hg-195

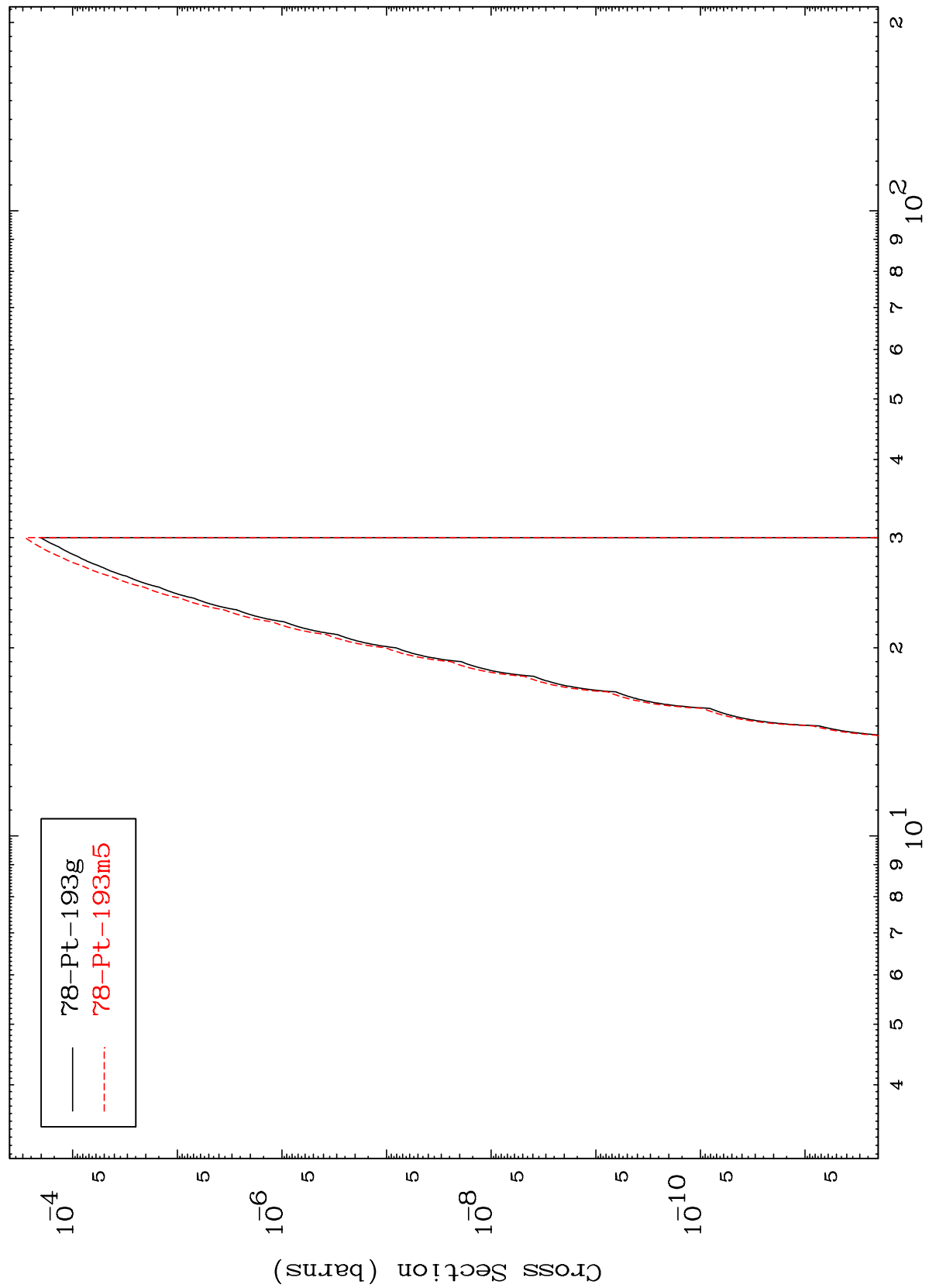
Incident Energy (MeV)

32

MAT 8023

80-Hg-195

Radionuclide Production Cross Section  
(n,He-3)



33

80-Hg-195

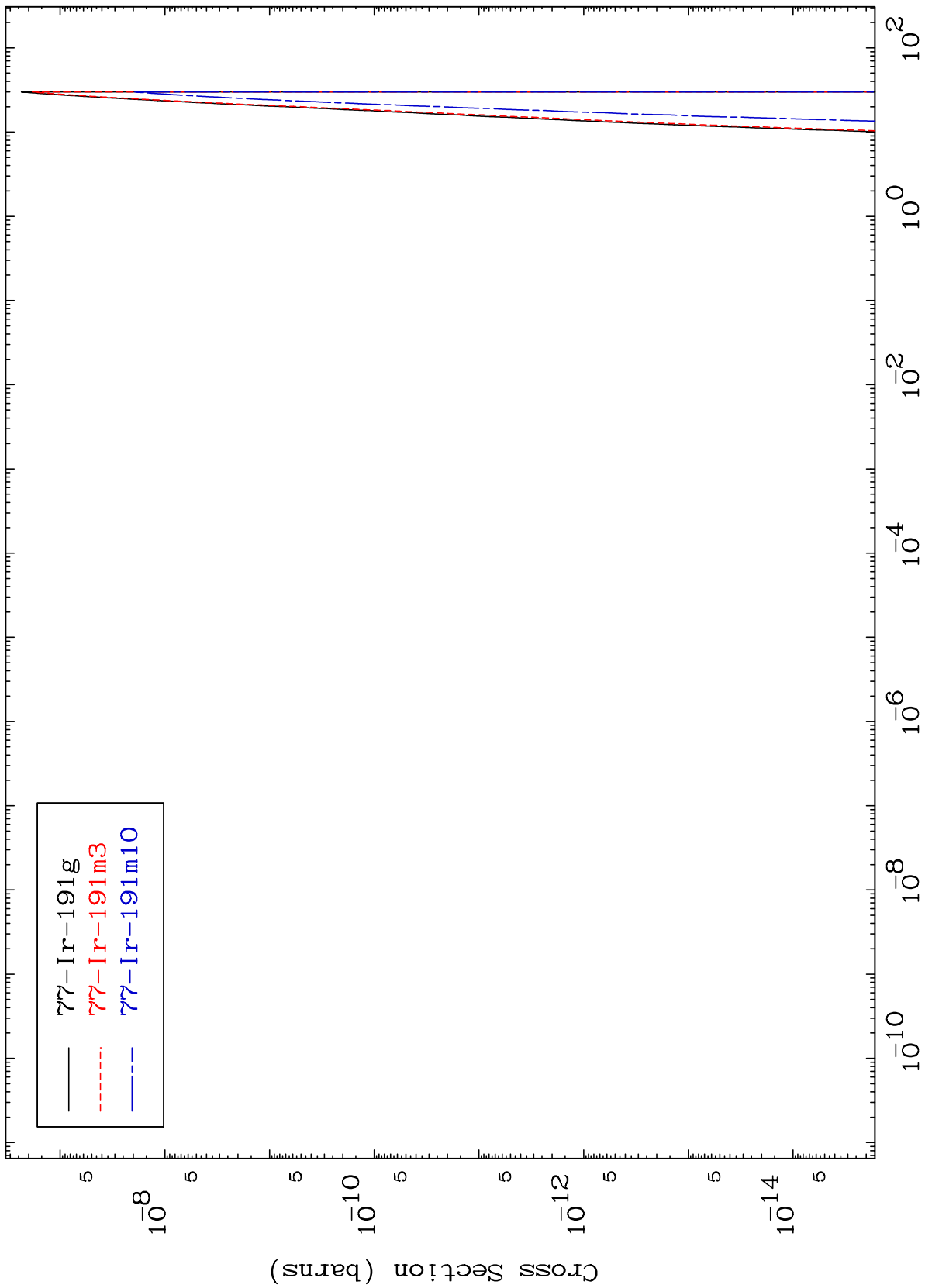
Incident Energy (MeV)

MAT 8023

(n,p)  $\alpha$

80-Hg-195

Radionuclide Production Cross Section

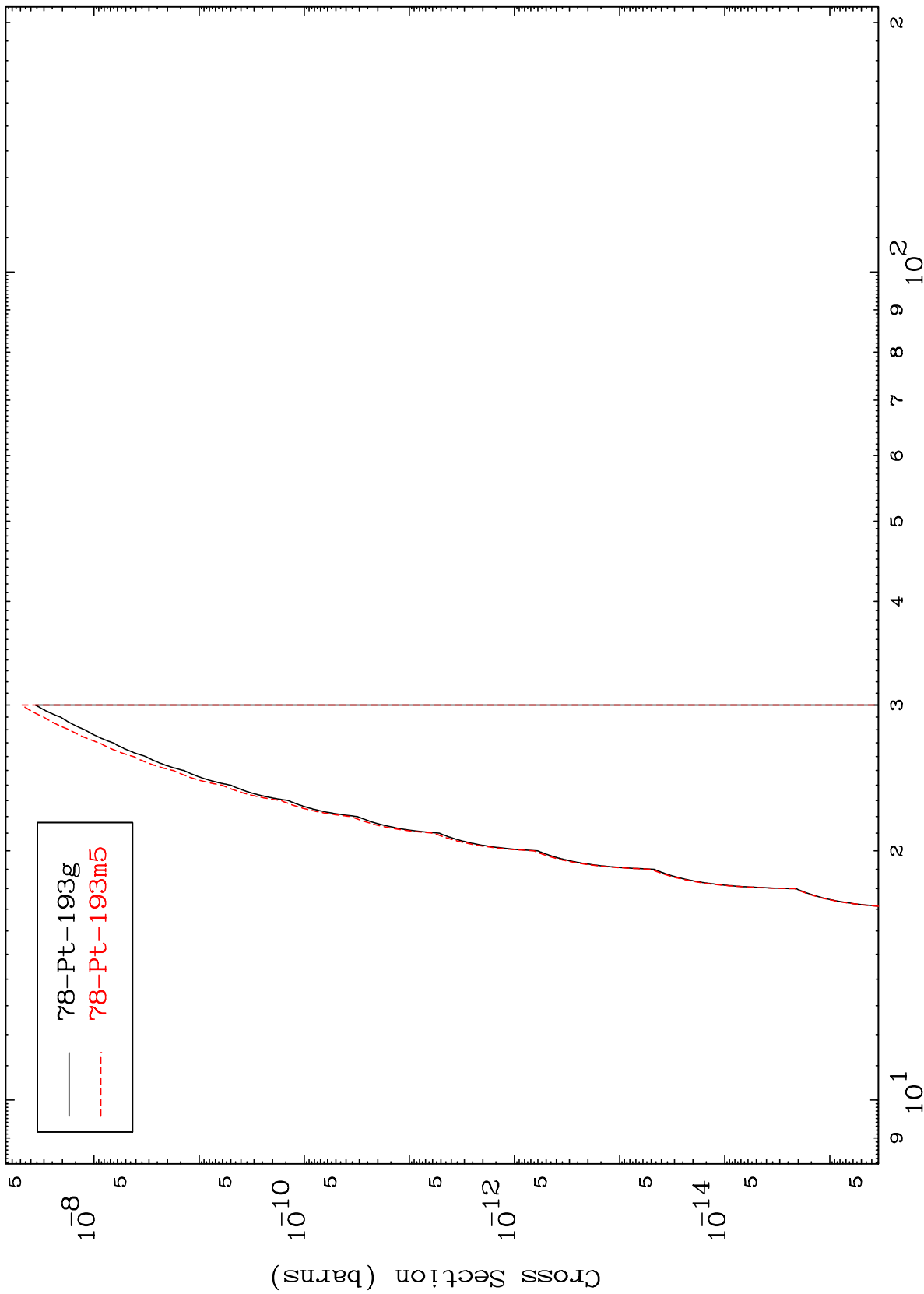


MAT 8023

(n,p) d

80-Hg-195

Radionuclide Production Cross Section



35

Incident Energy (MeV)

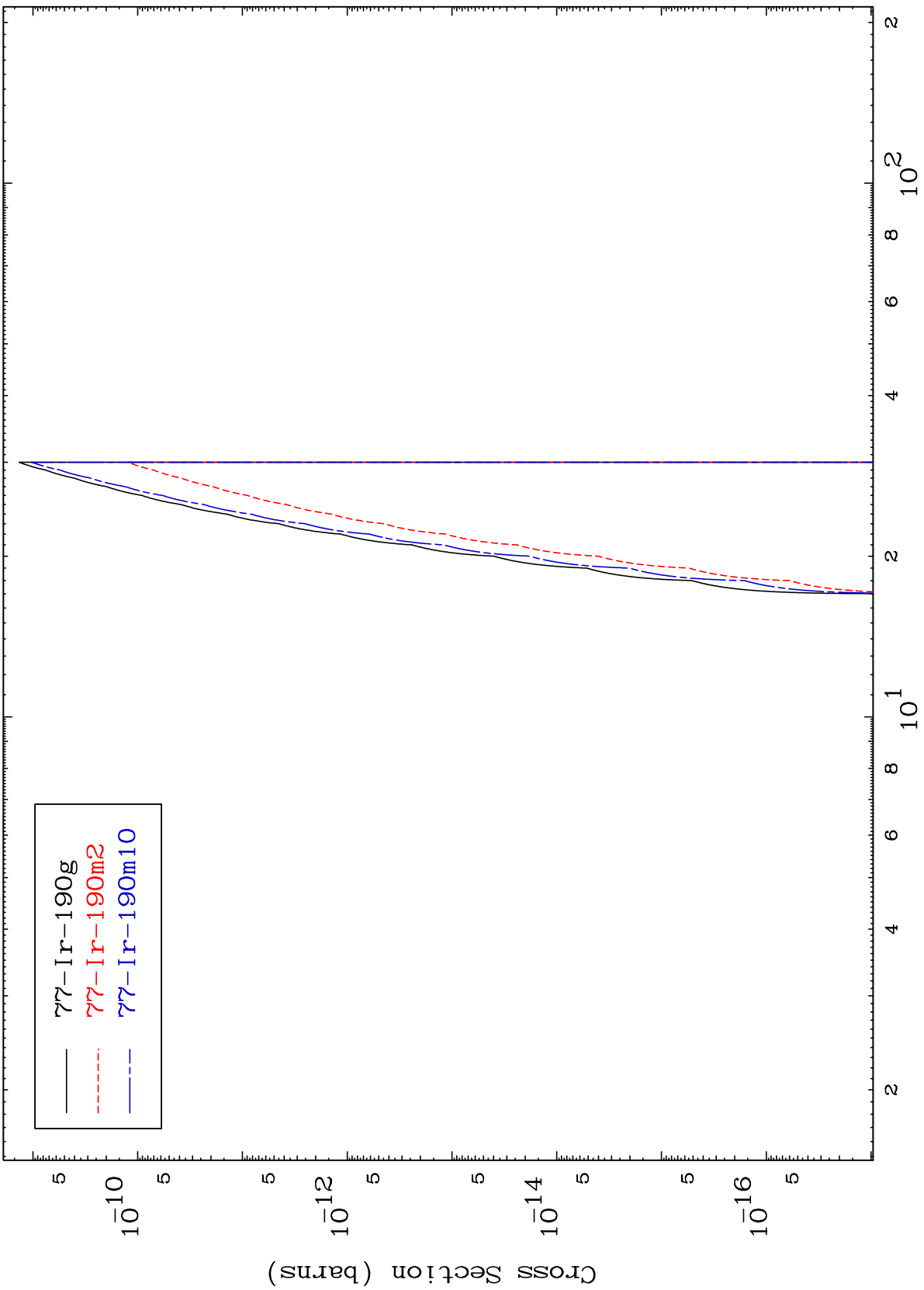
80-Hg-195

MAT 8023

(n,d)  $\alpha$

80-Hg-195

Radionuclide Production Cross Section



36

Incident Energy (MeV)

80-Hg-195