

Program EVALPLOT
(Version 2018-1)

by

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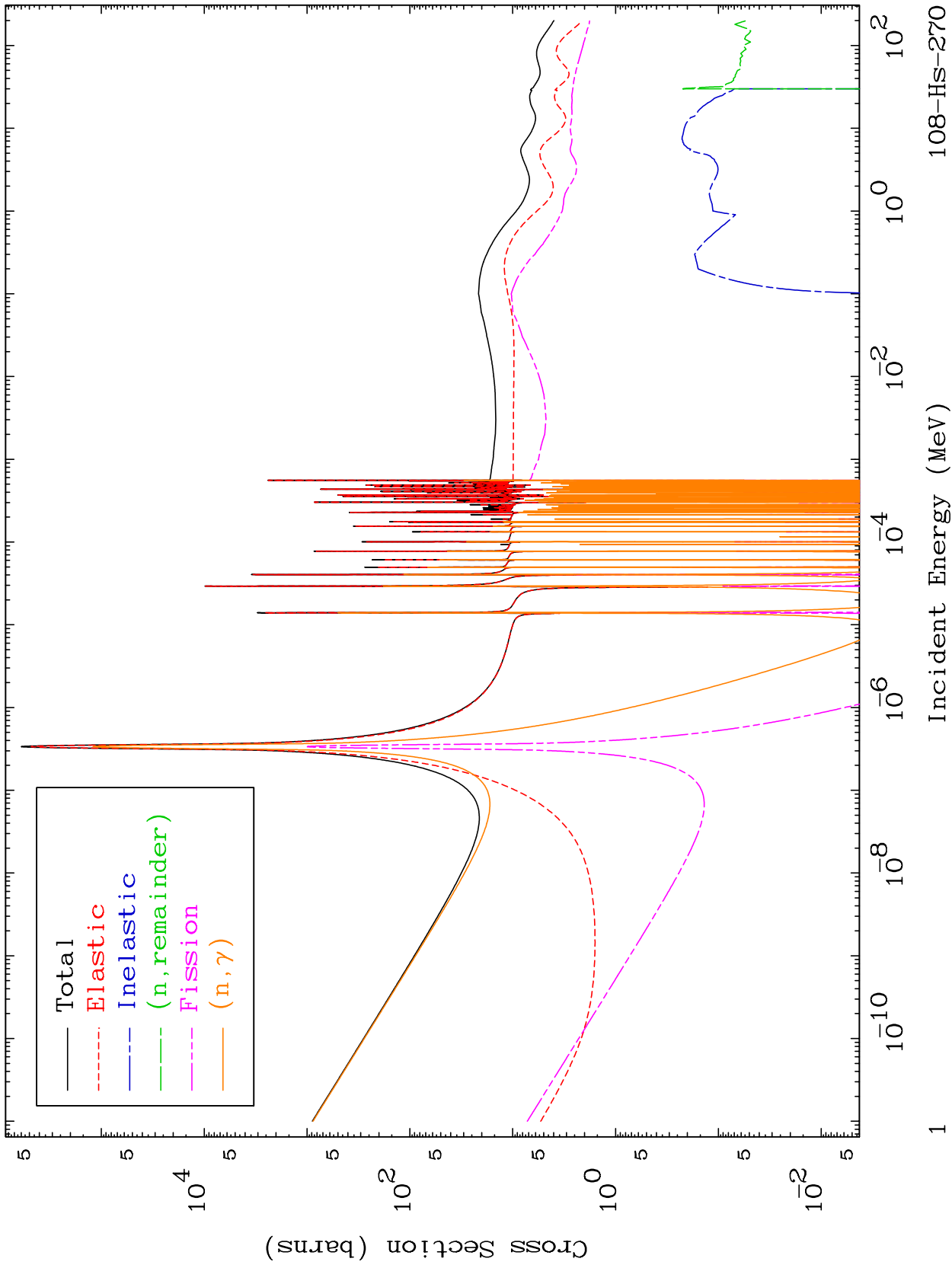
Press Mouse Button to Start

MAT 870

Major

293 Kelvin Cross Sections

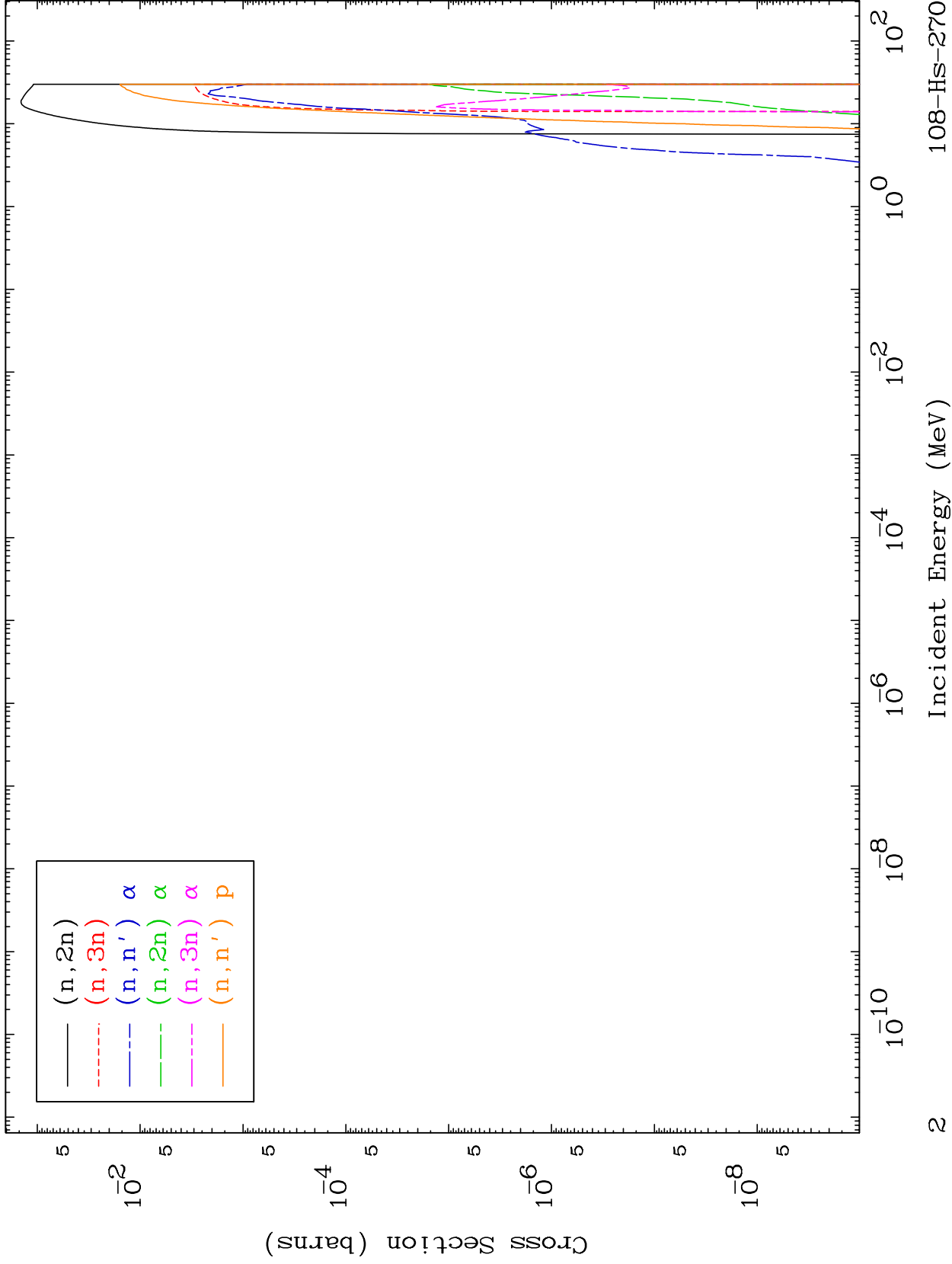
108-Hs-270



MAT 870

Neutron Production
293 Kelvin Cross Sections

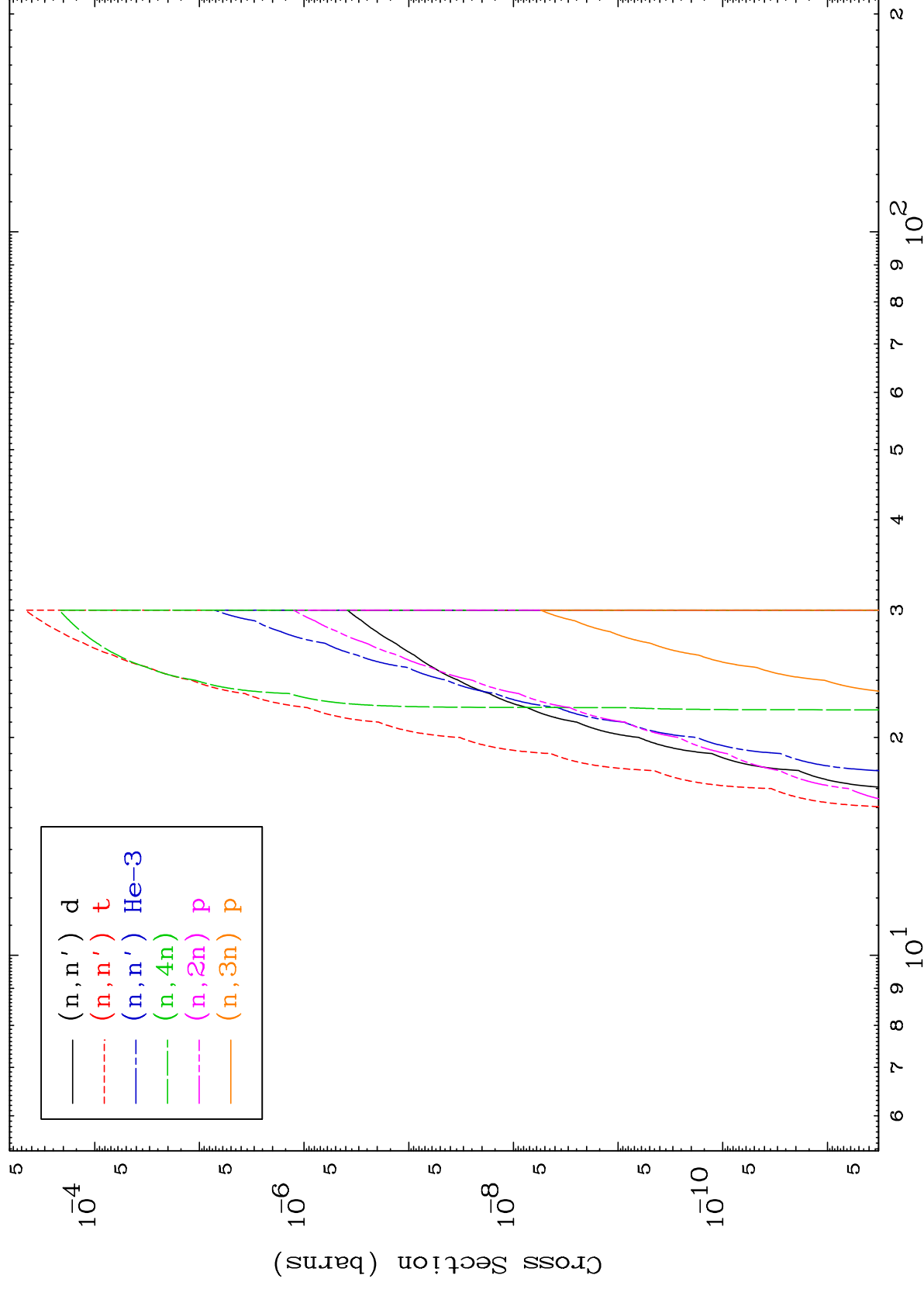
108-Hs-270



MAT 870

Neutron Production
293 Kelvin Cross Sections

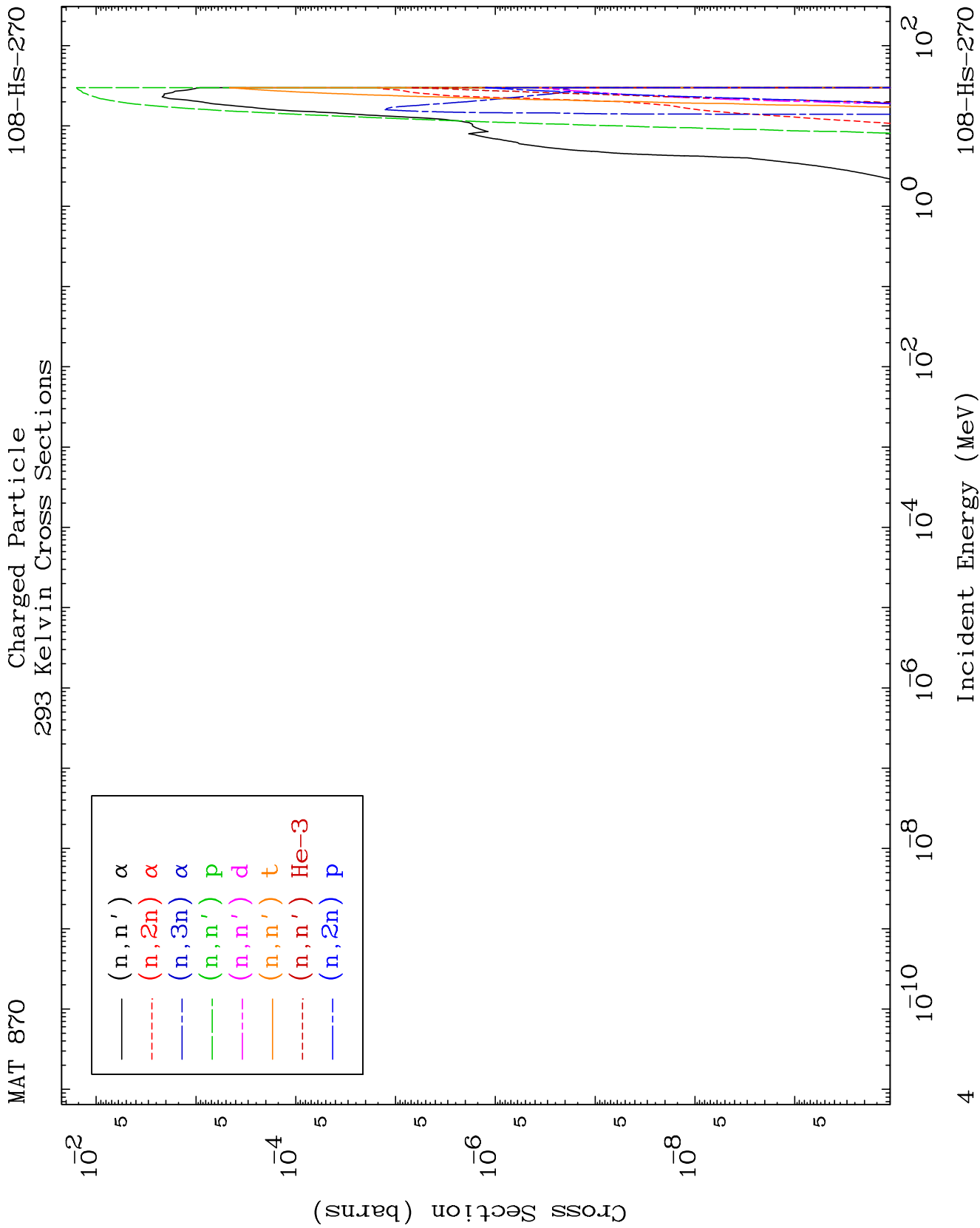
108-Hs-270

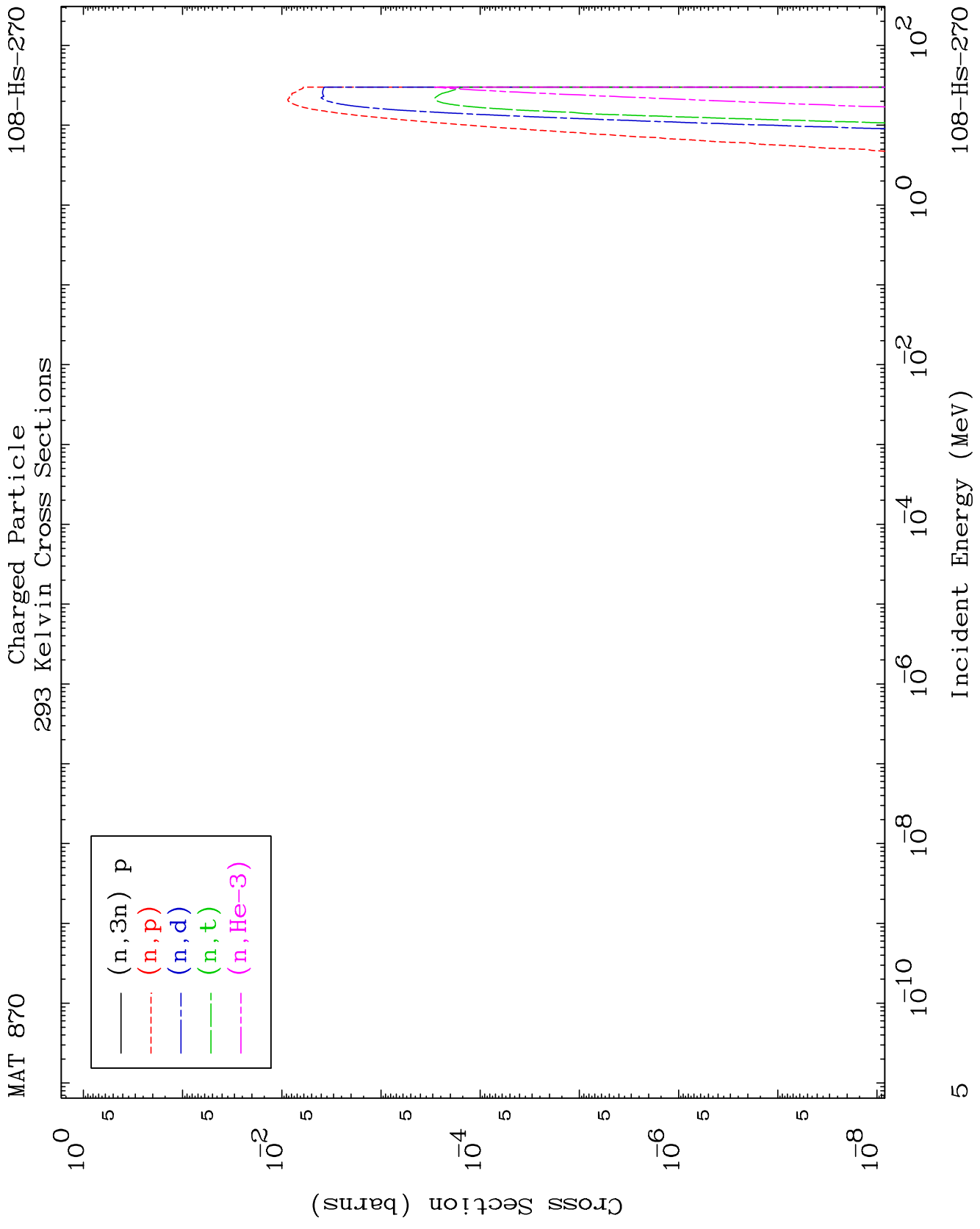


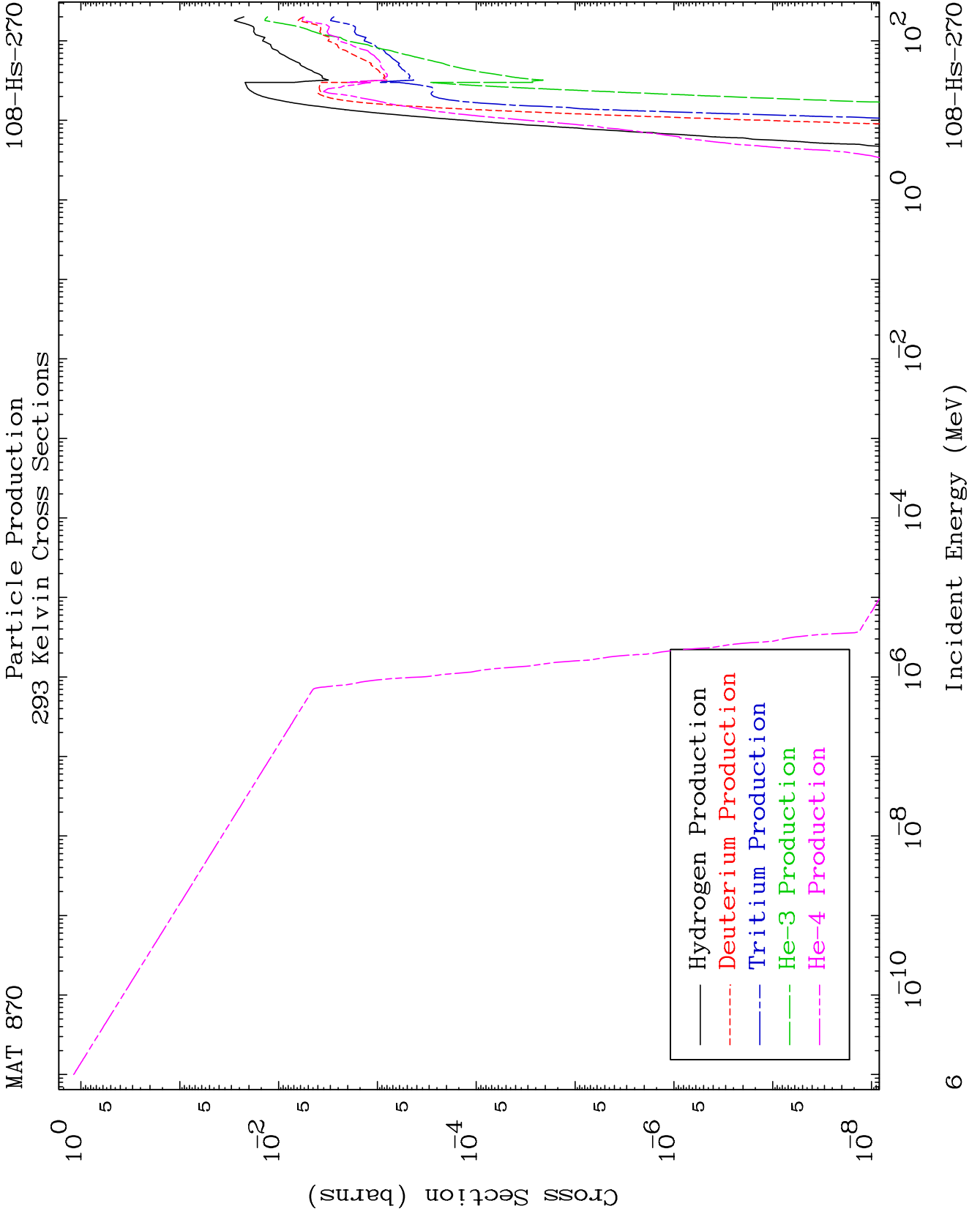
3

Incident Energy (MeV)

108-Hs-270



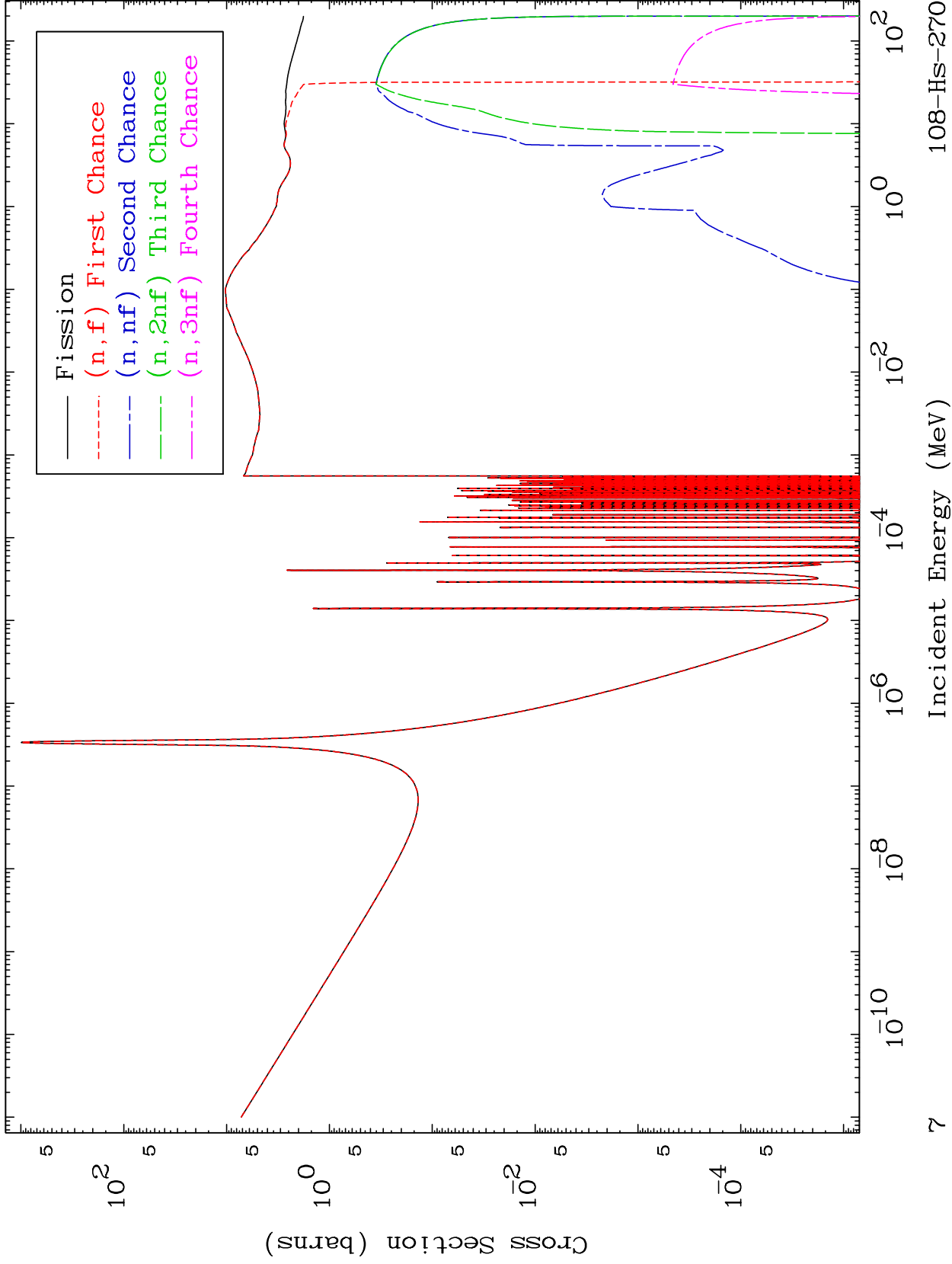




MAT 870

Fission
293 Kelvin Cross Sections

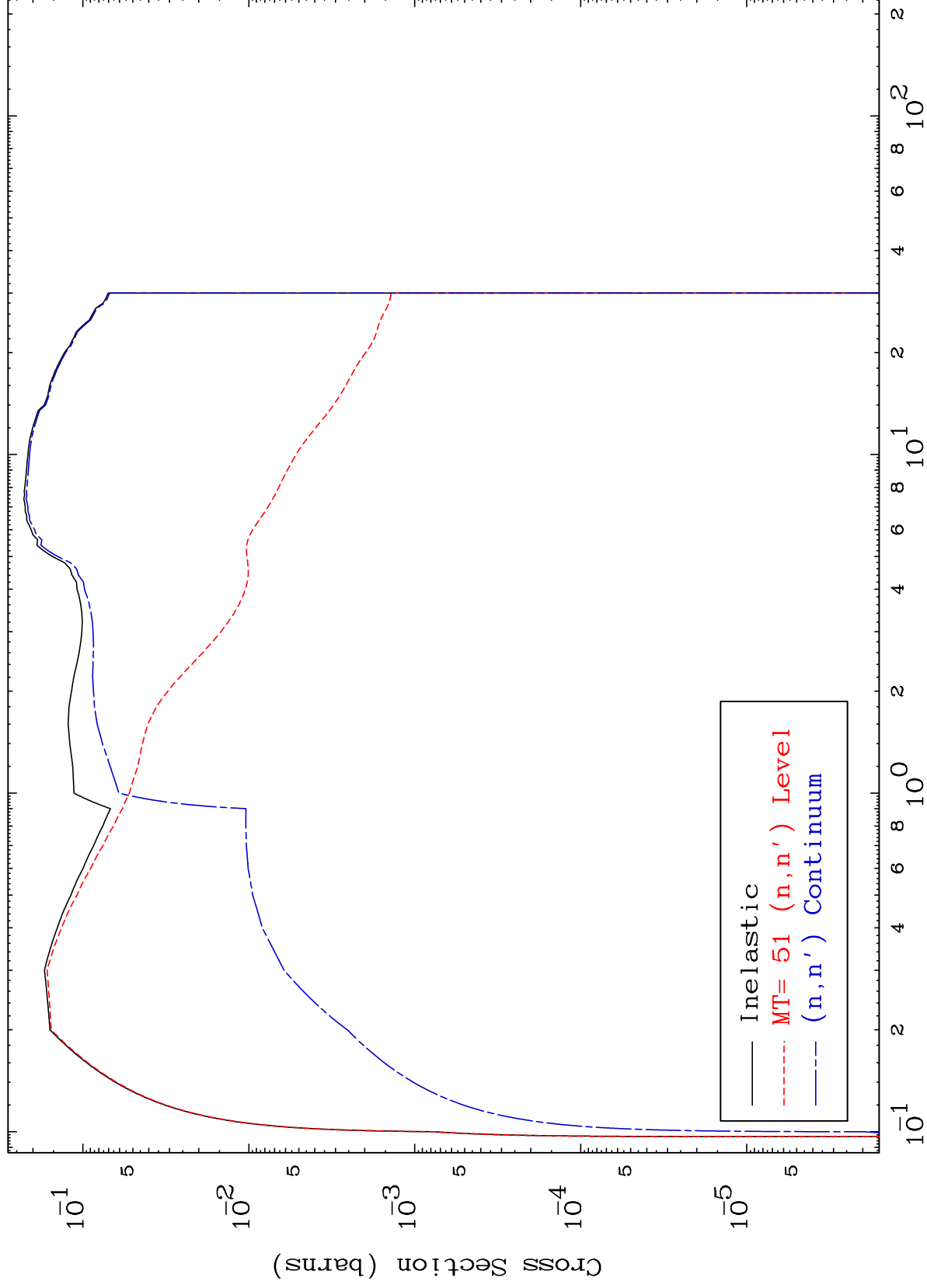
108-Hs-270



MAT 870

(n,n') Level
293 Kelvin Cross Sections

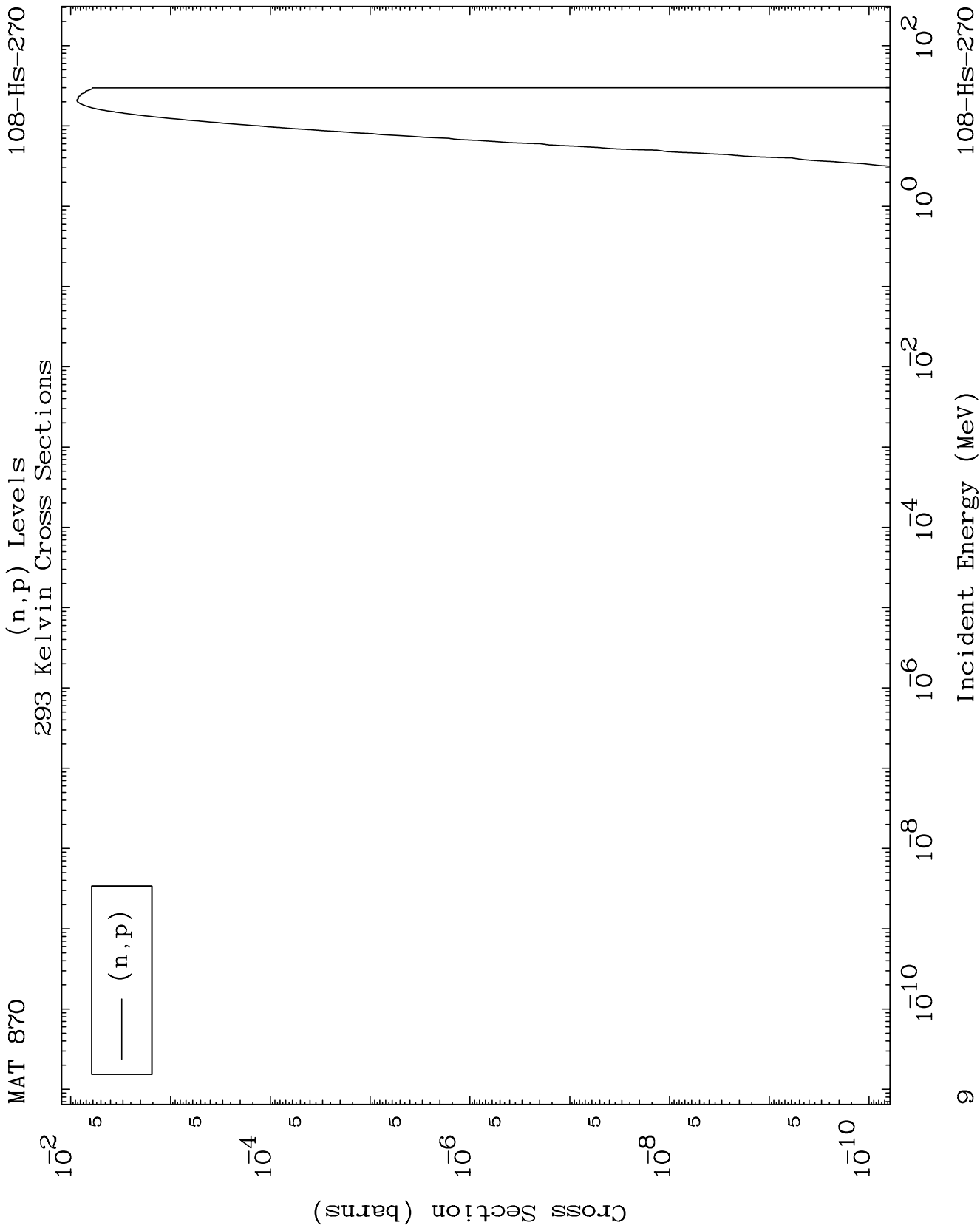
108-Hs-270



Incident Energy (MeV)

108-Hs-270

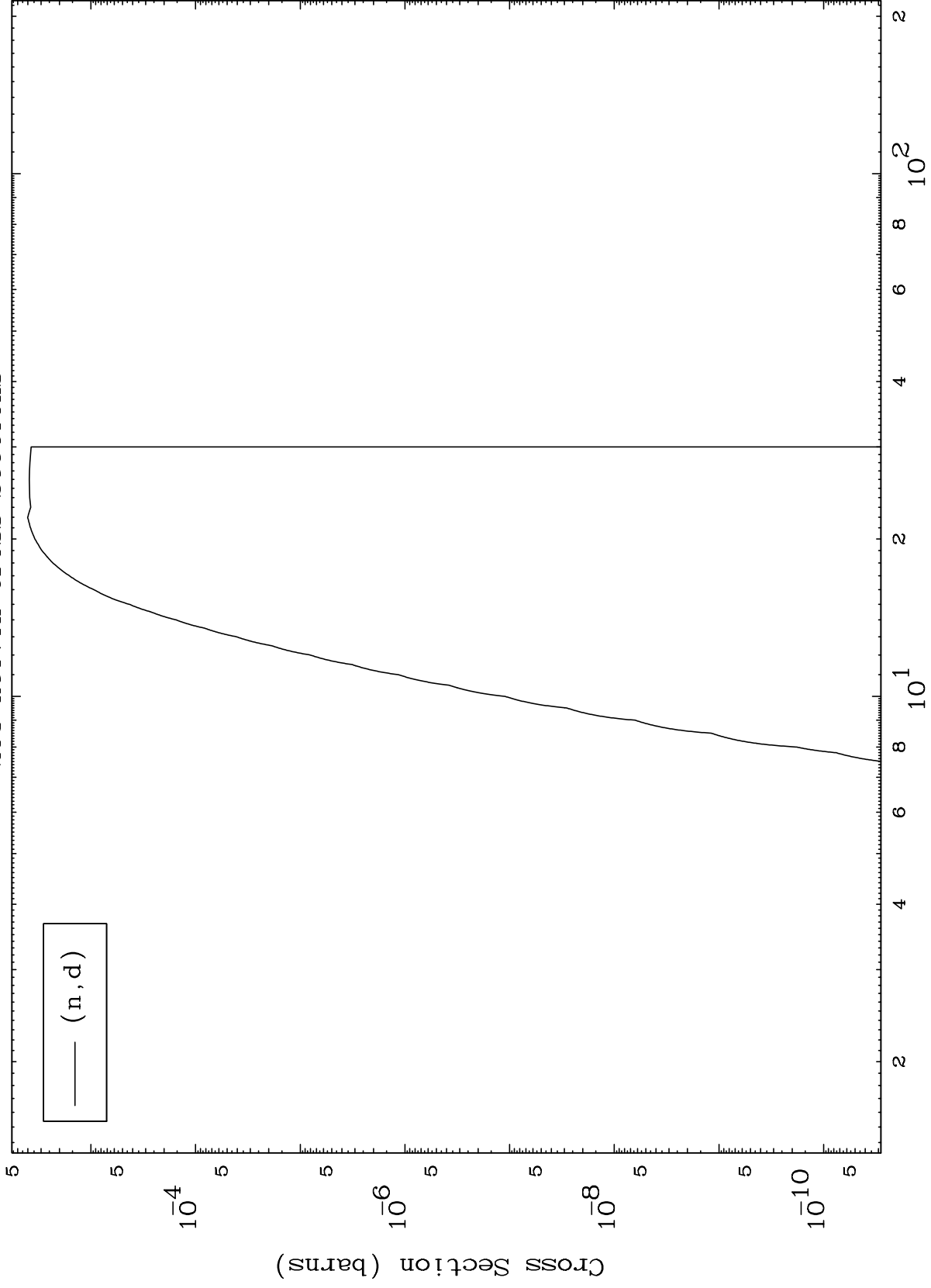
8



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(n,d) Levels
293 Kelvin Cross Sections

108-Hs-270



10

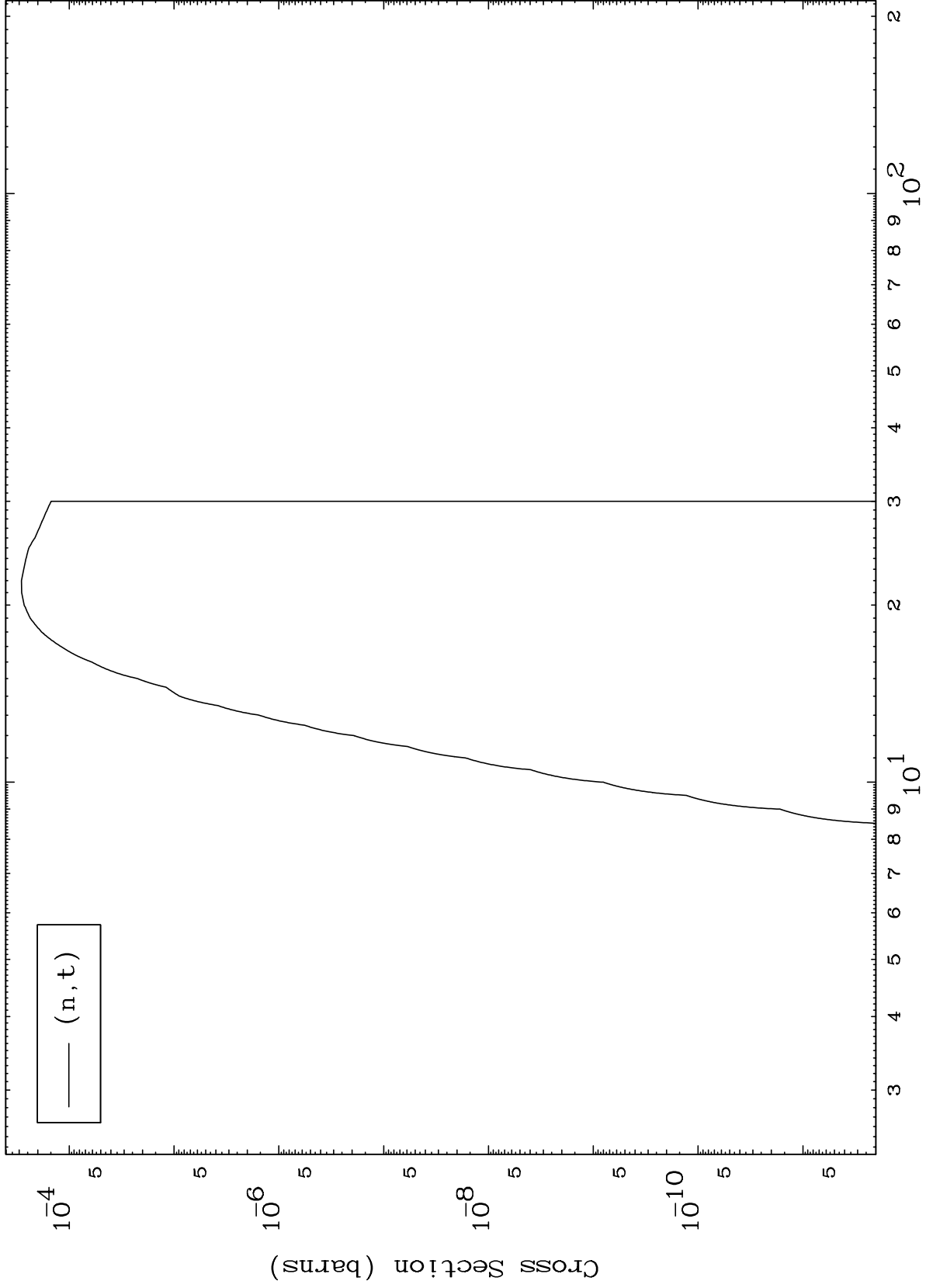
Incident Energy (MeV)

108-Hs-270

MAT 870

(n,t) Levels
293 Kelvin Cross Sections

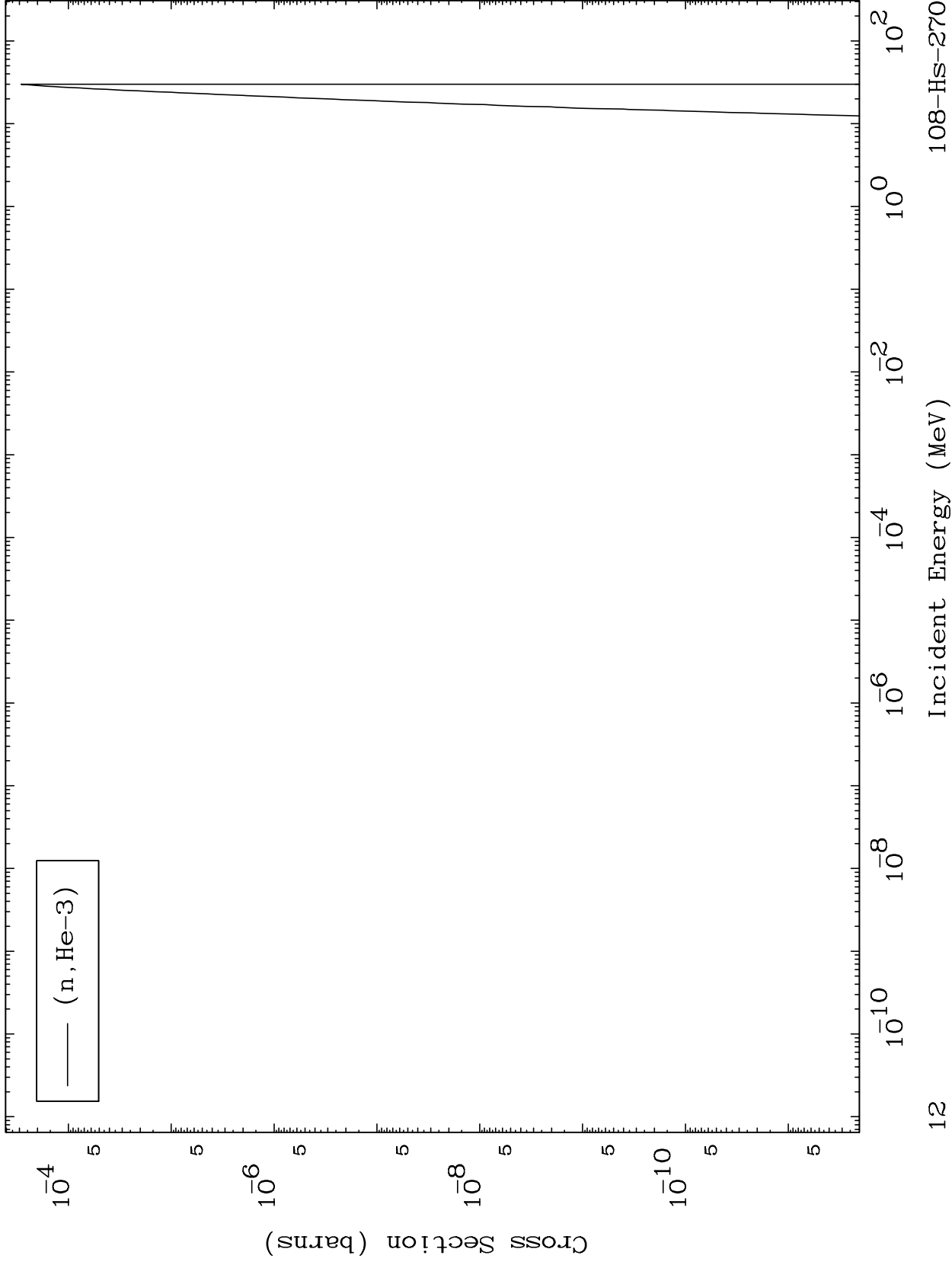
108-Hs-270

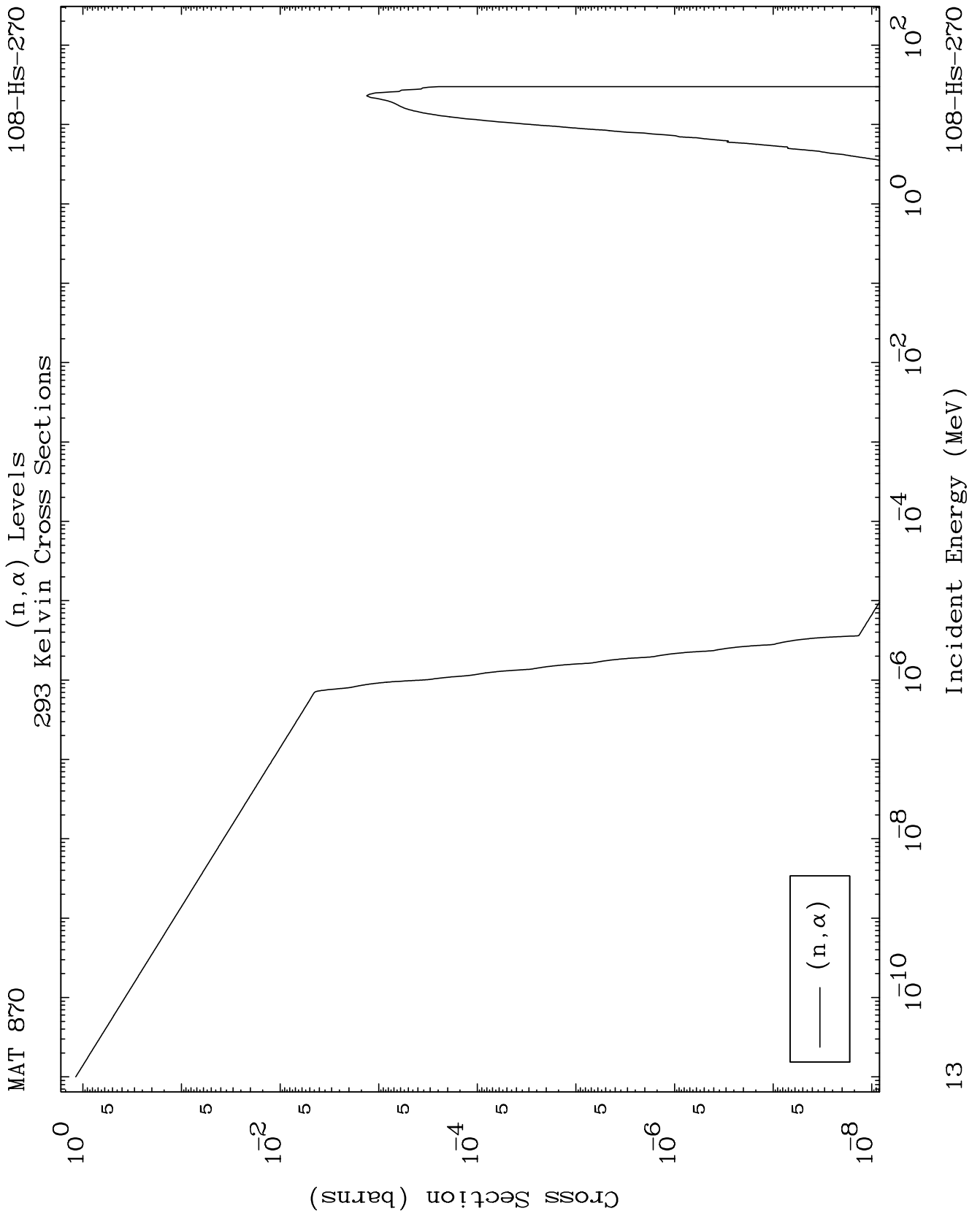


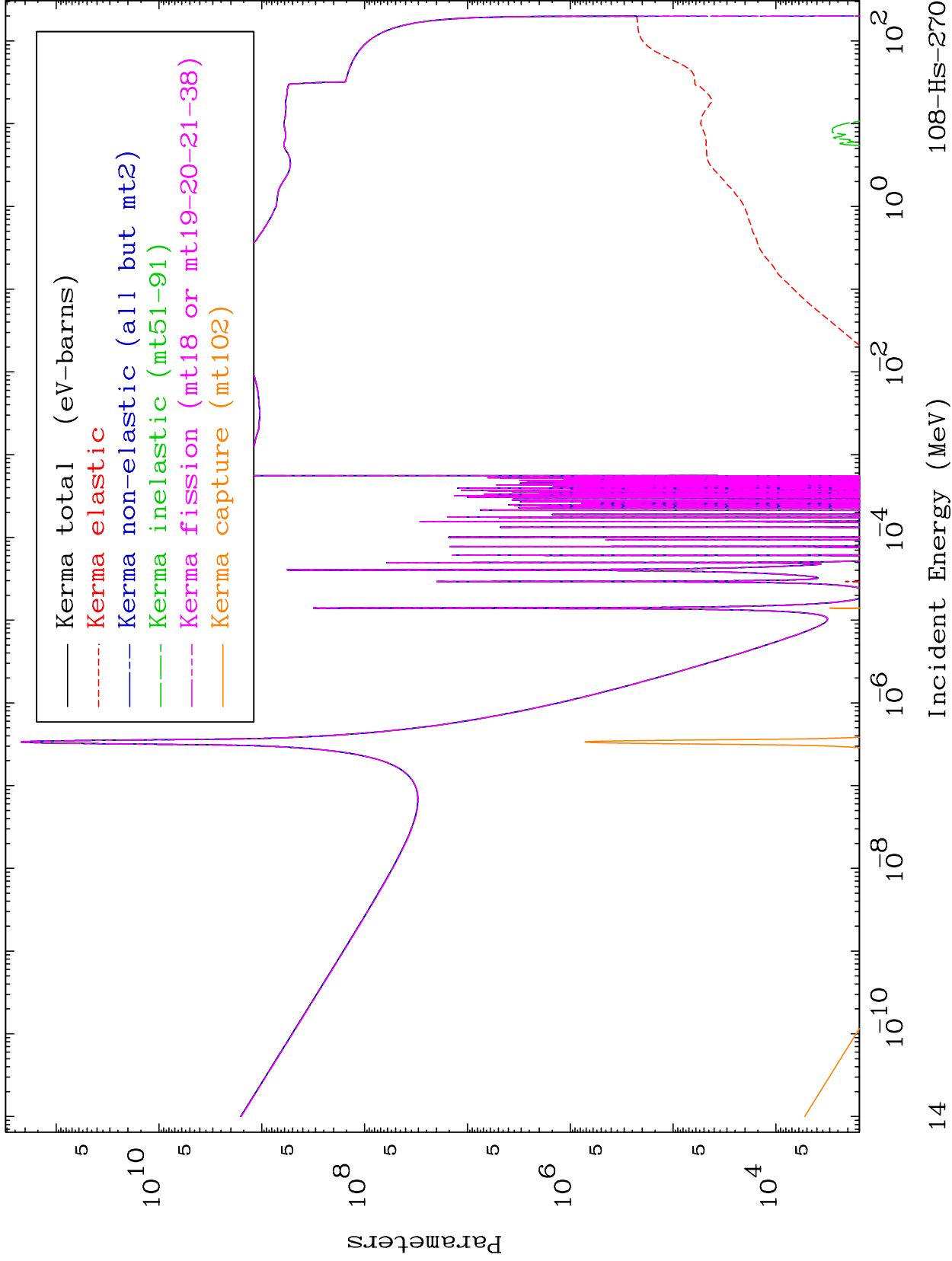
MAT 870

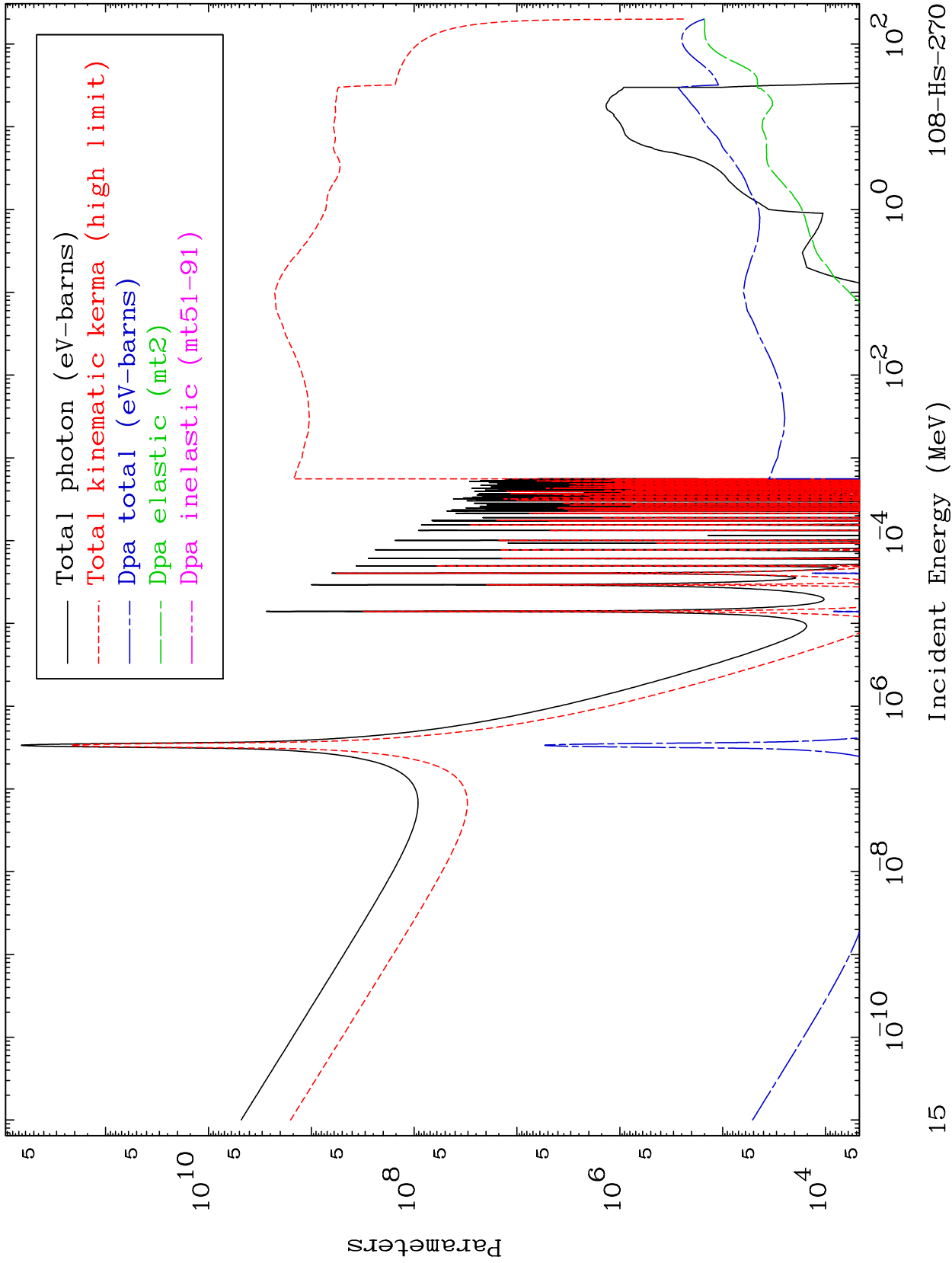
(n,He3) Levels
293 Kelvin Cross Sections

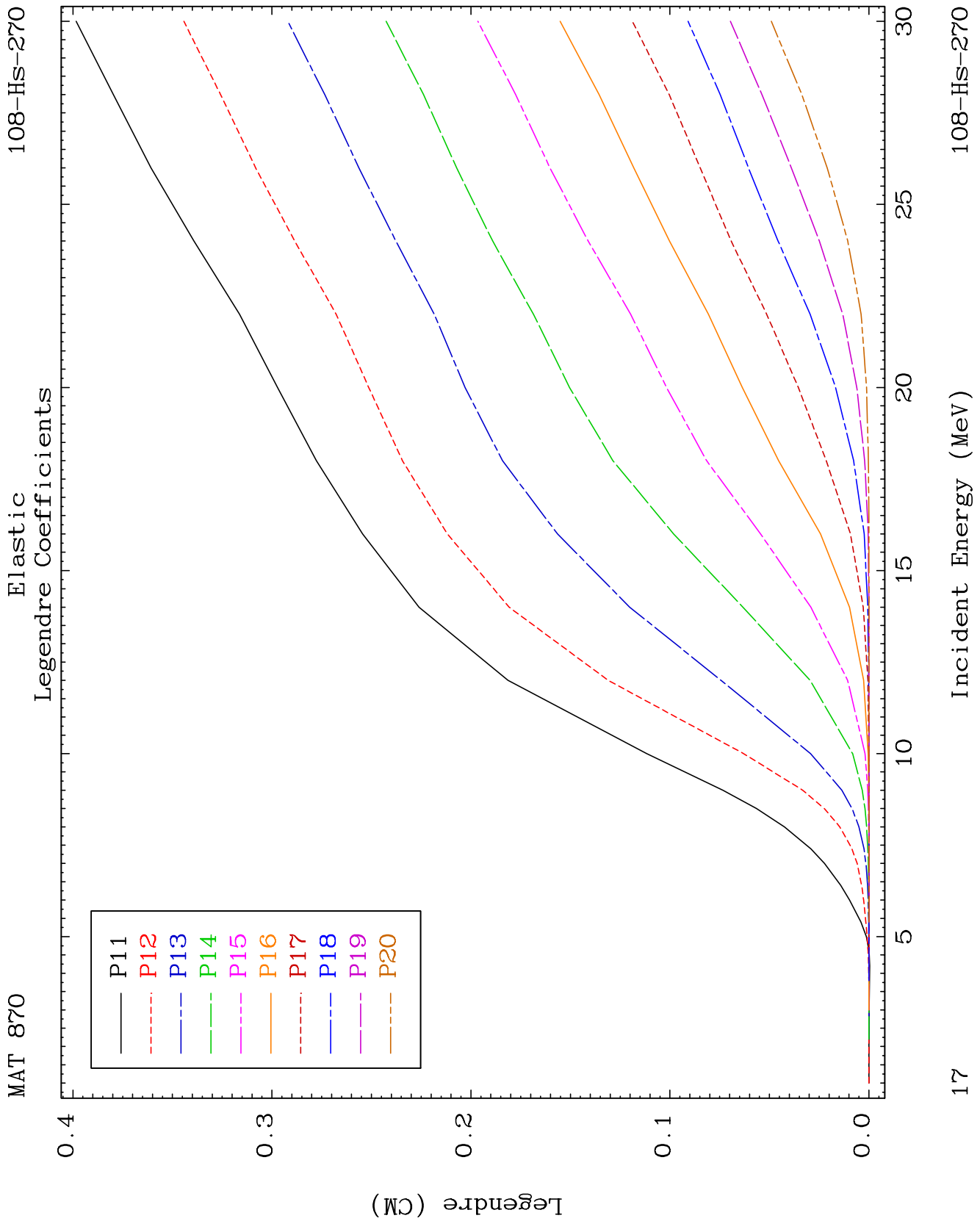
108-Hs-270







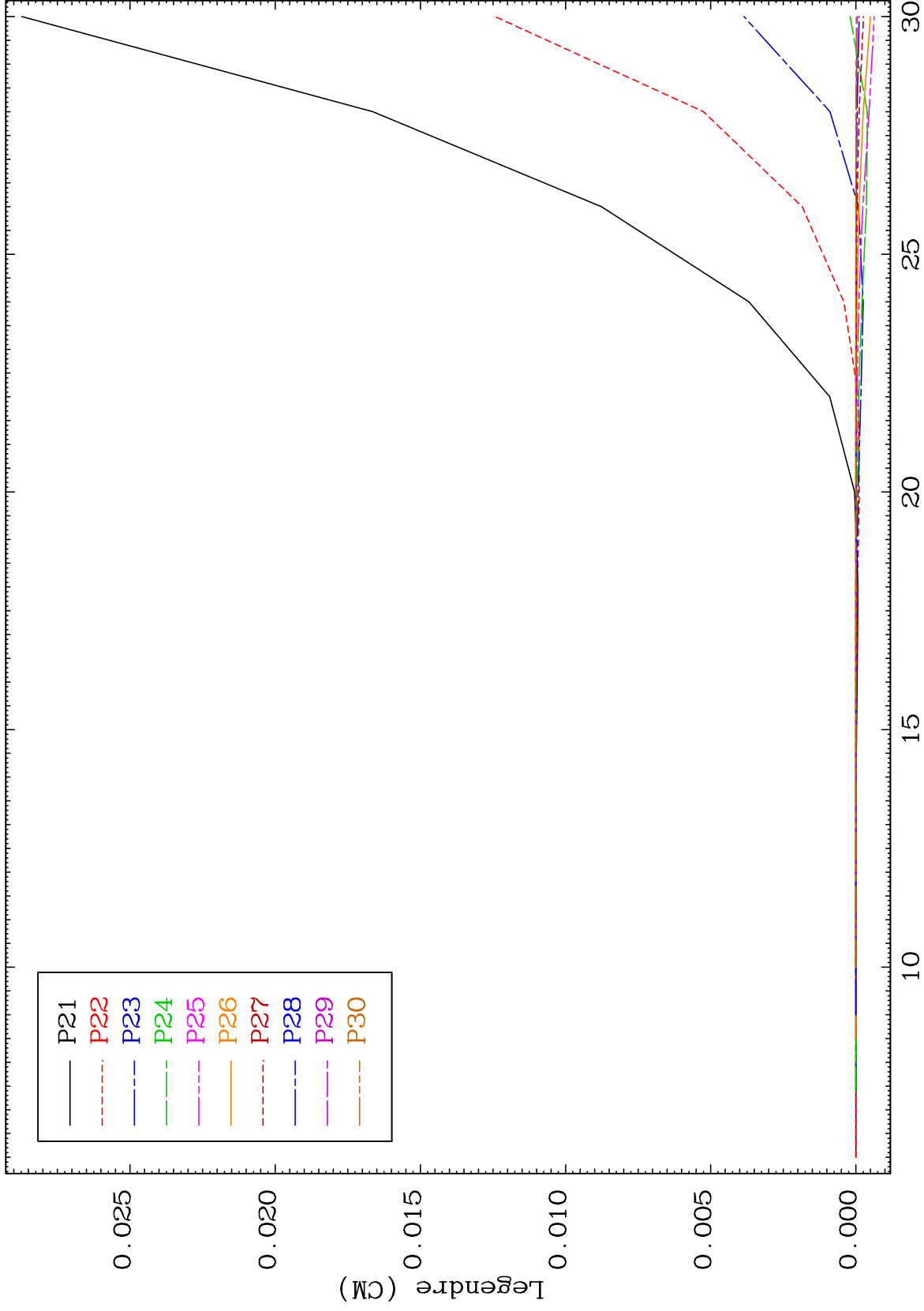




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Elastic Legendre Coefficients

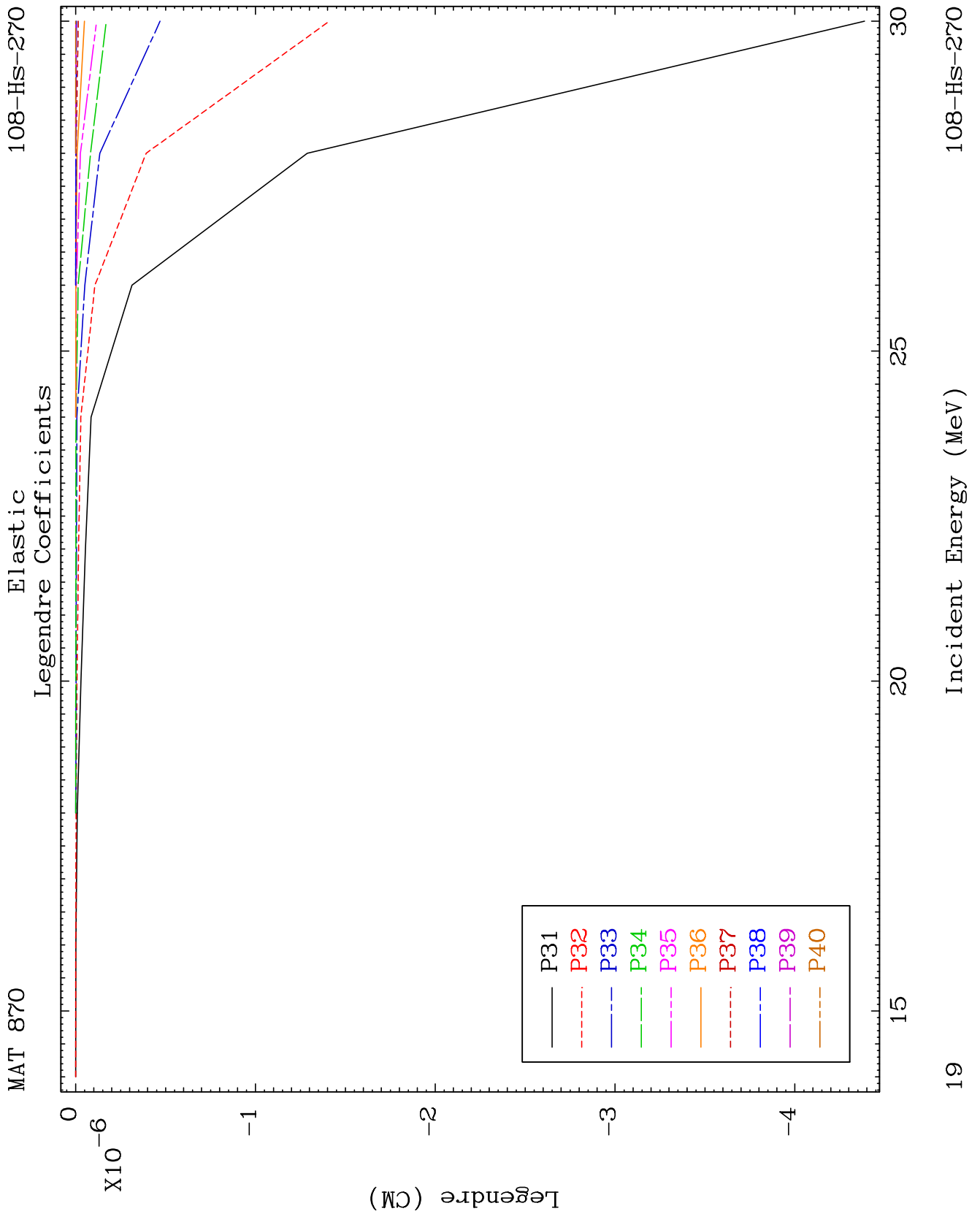
108-Hs-270



18

Incident Energy (MeV)

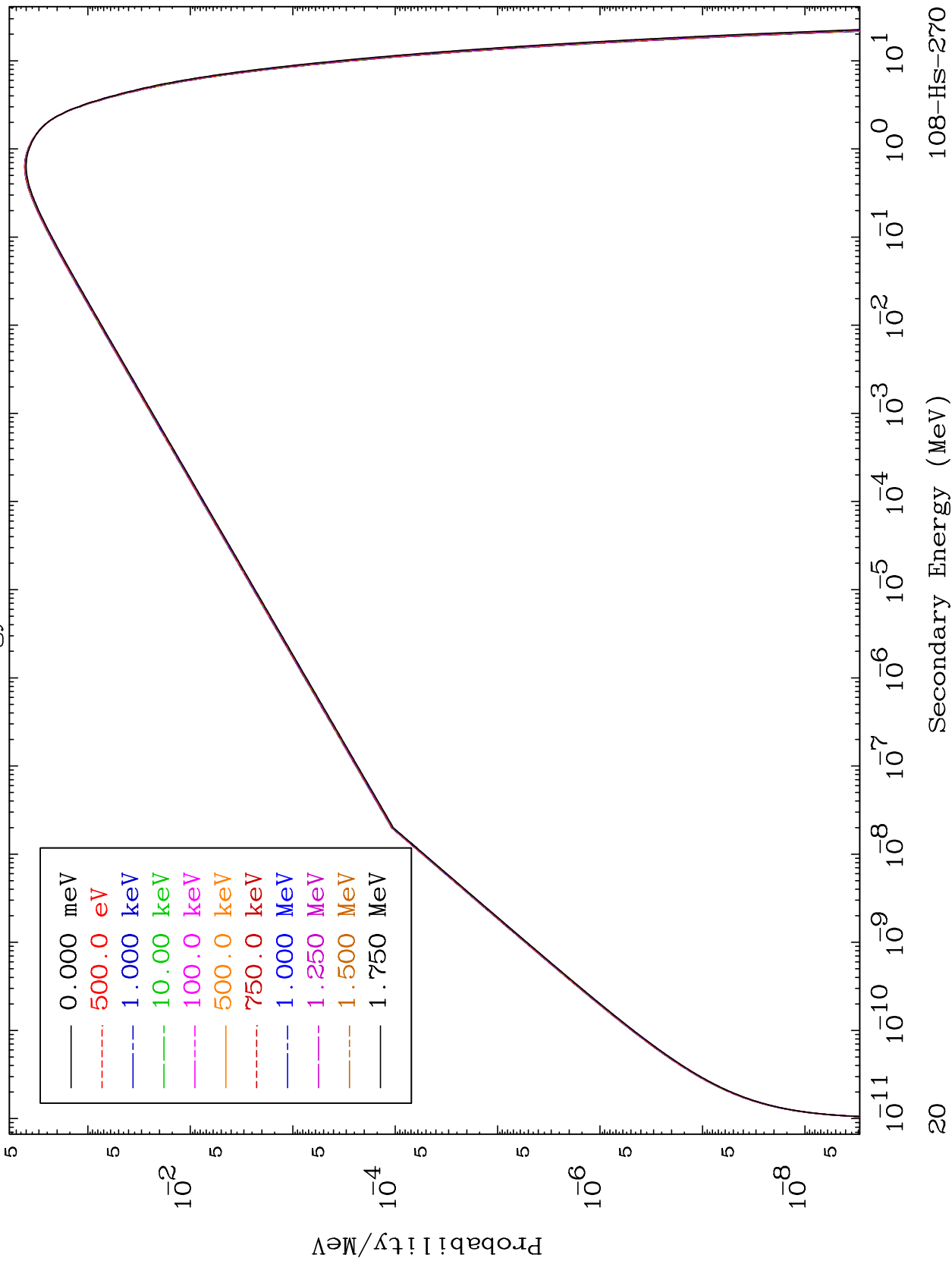
108-Hs-270



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Fission Energy Distribution

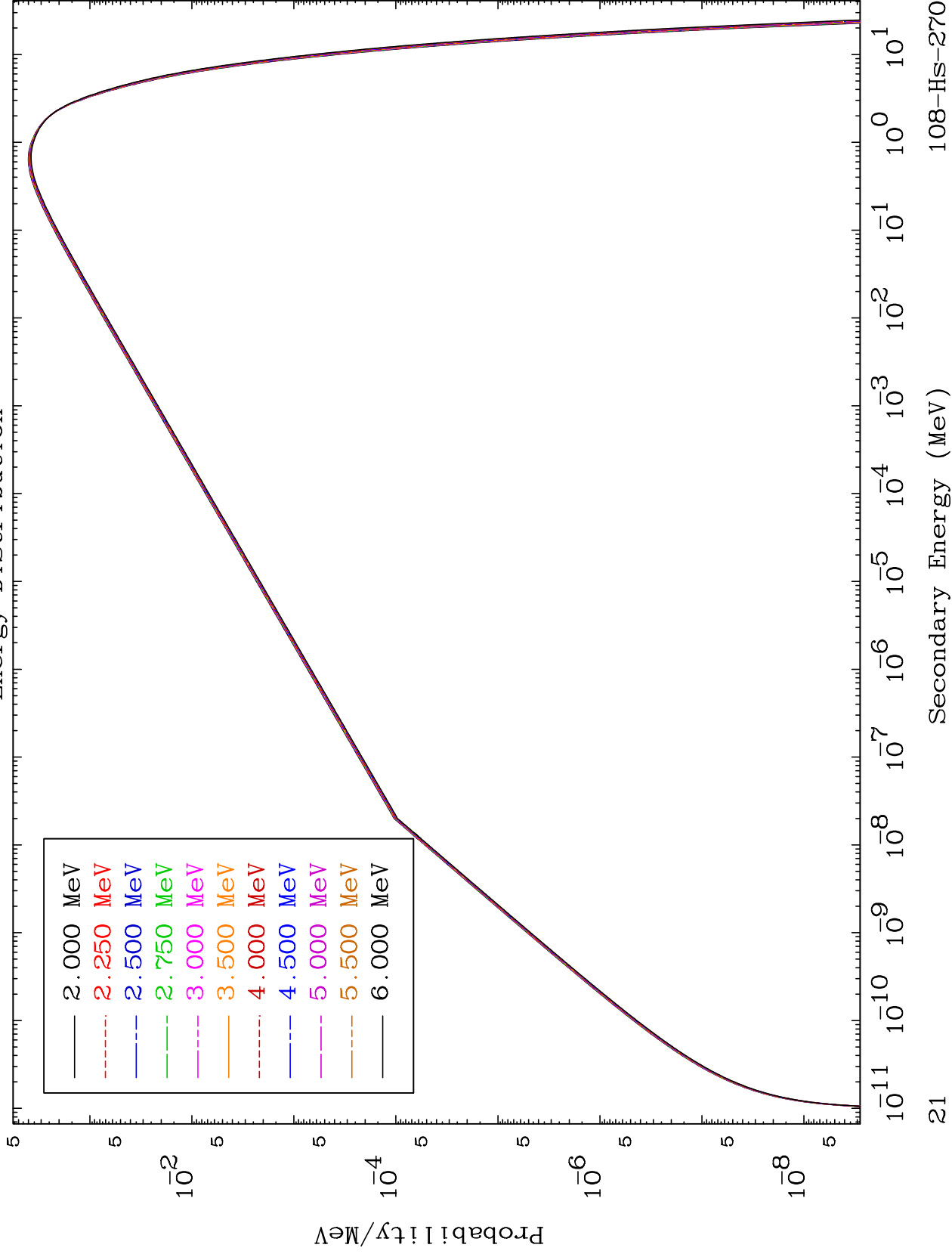
108-Hs-270



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Fission Energy Distribution

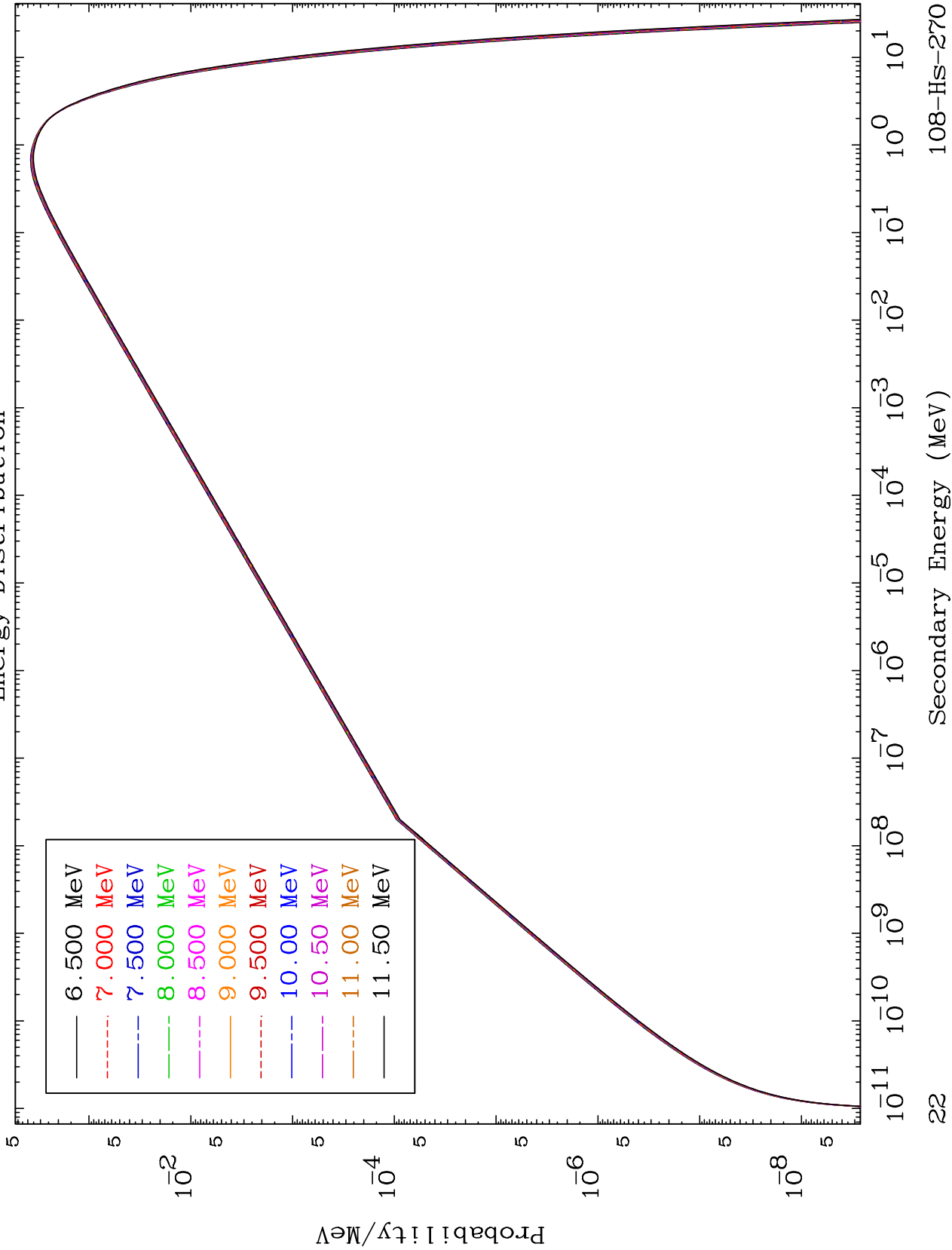
108-Hs-270



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Fission Energy Distribution

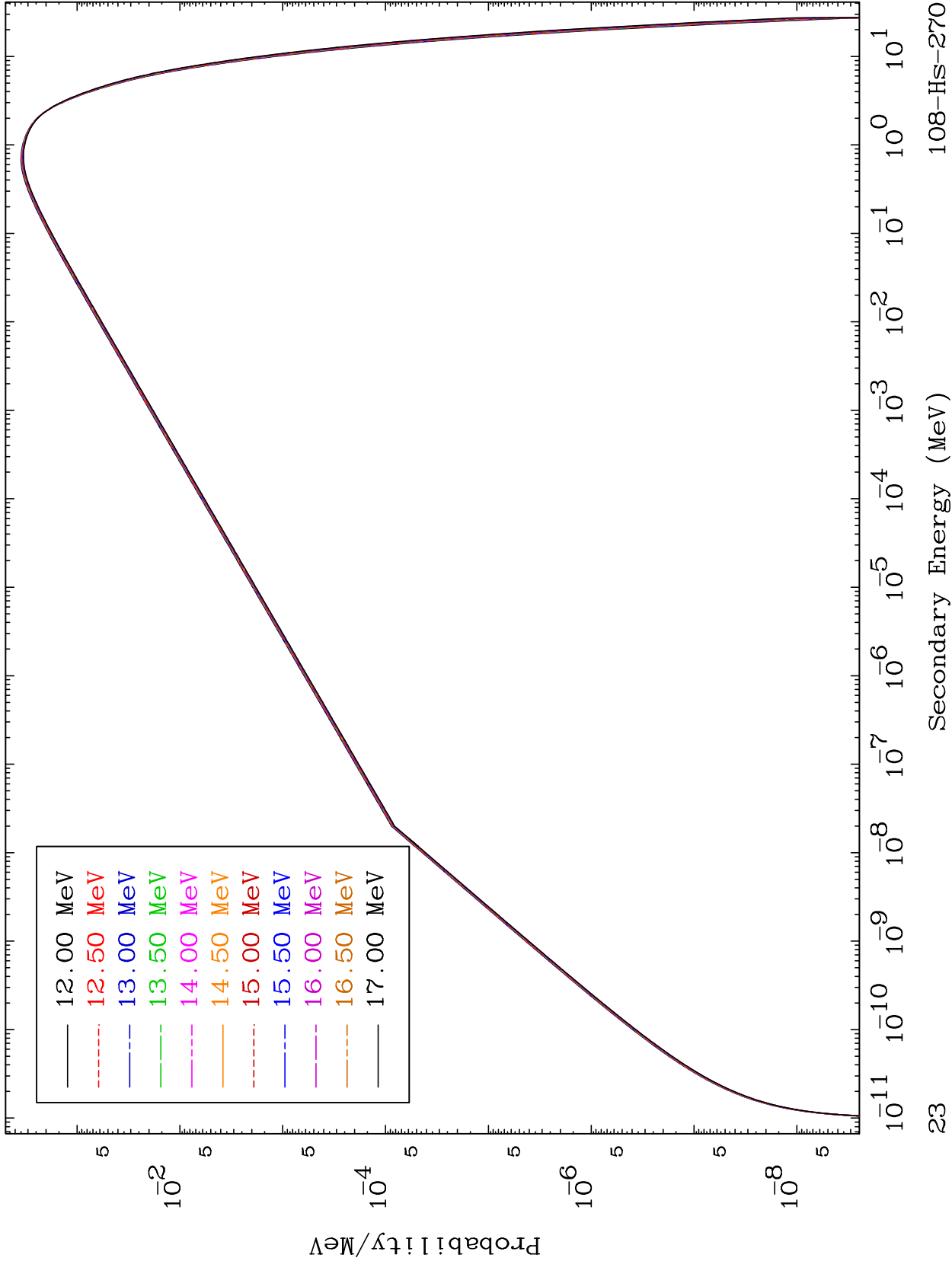
108-Hs-270



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Fission Energy Distribution

108-Hs-270

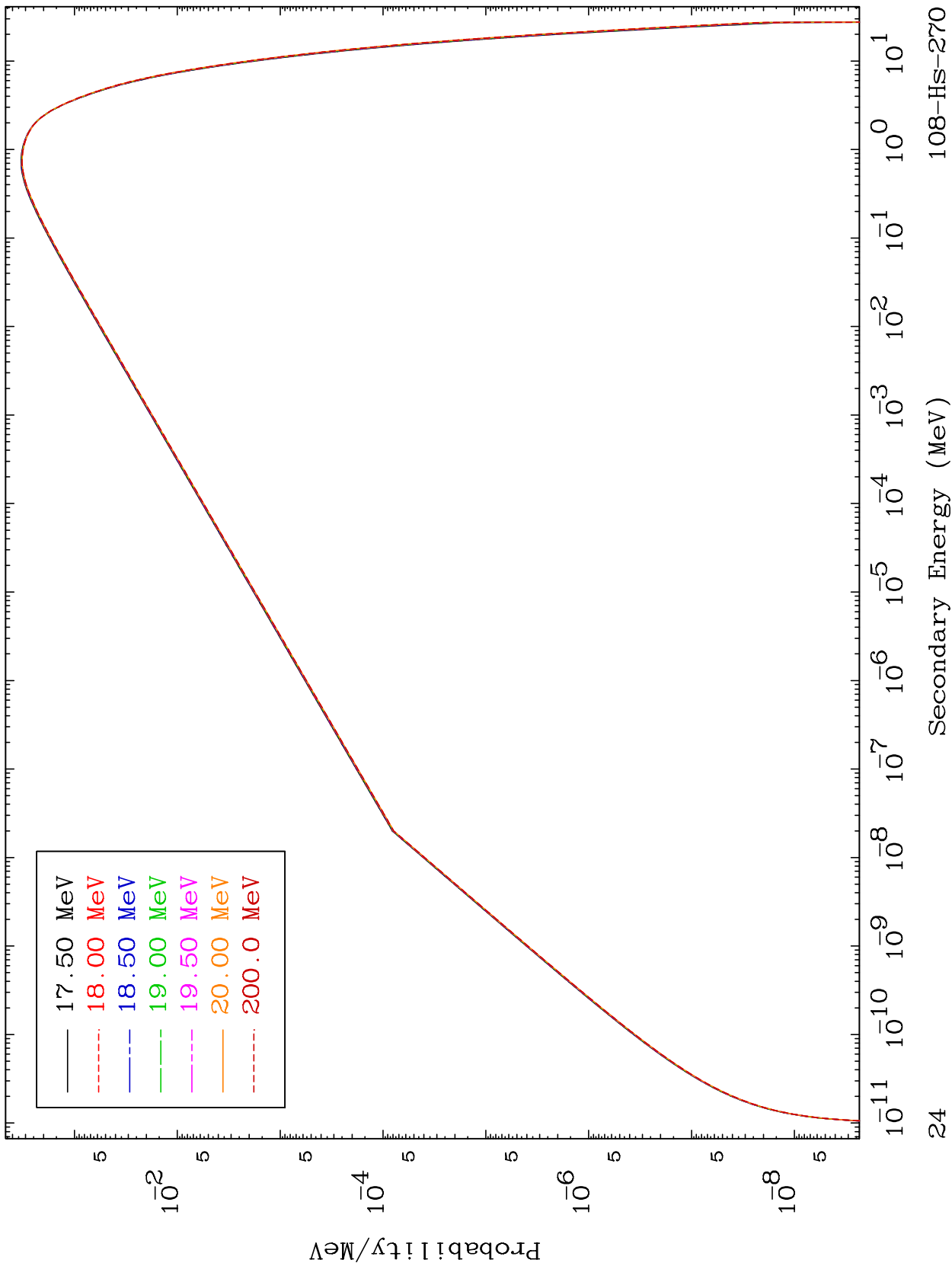


108-Hs-270

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Fission Energy Distribution

108-Hs-270

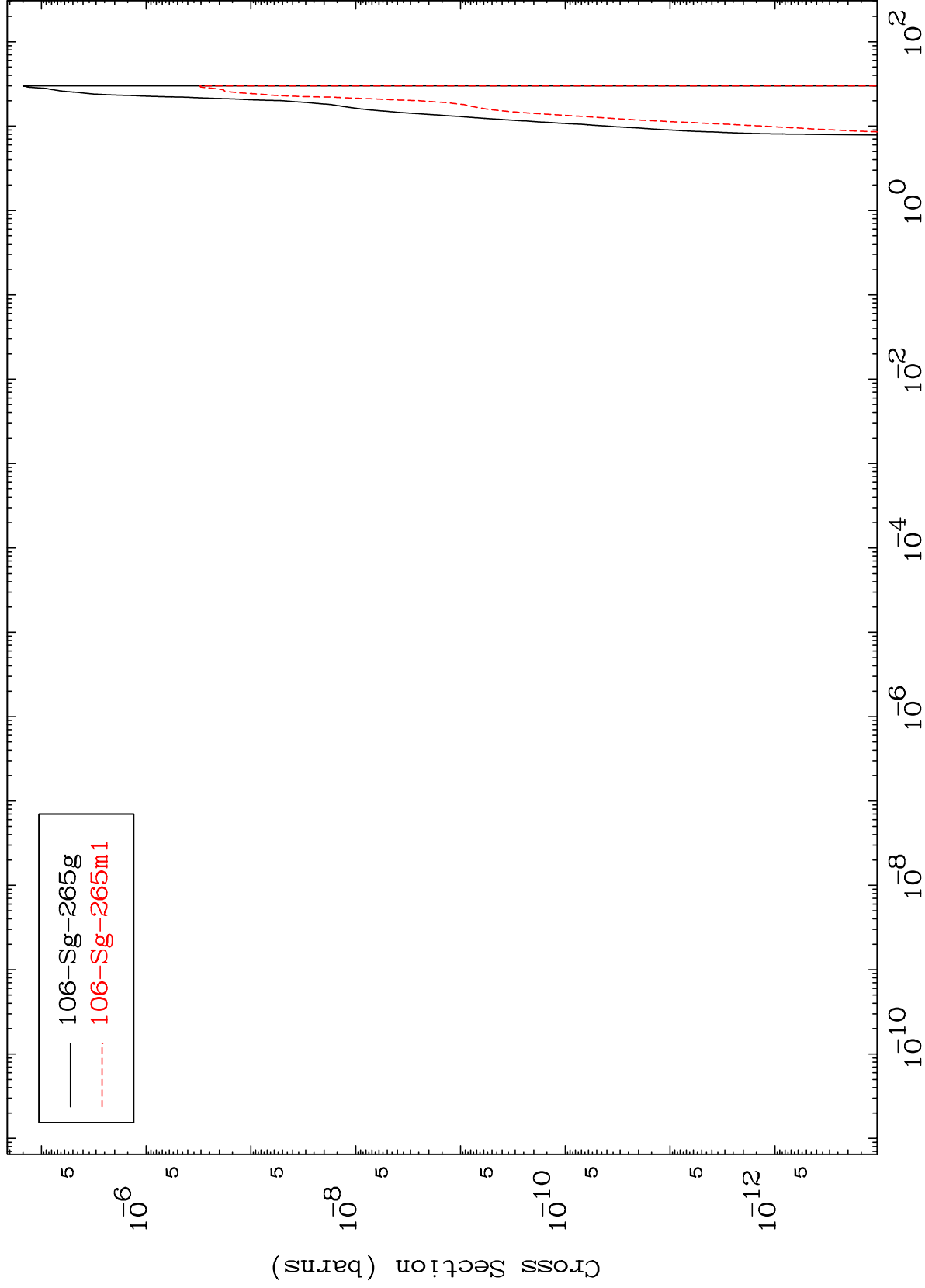


MAT 870

108-Hs-270

(n,2n) α

Radionuclide Production Cross Section



25

Incident Energy (MeV)

108-Hs-270