

Program EVALPLOT
(Version 2018-1)

by

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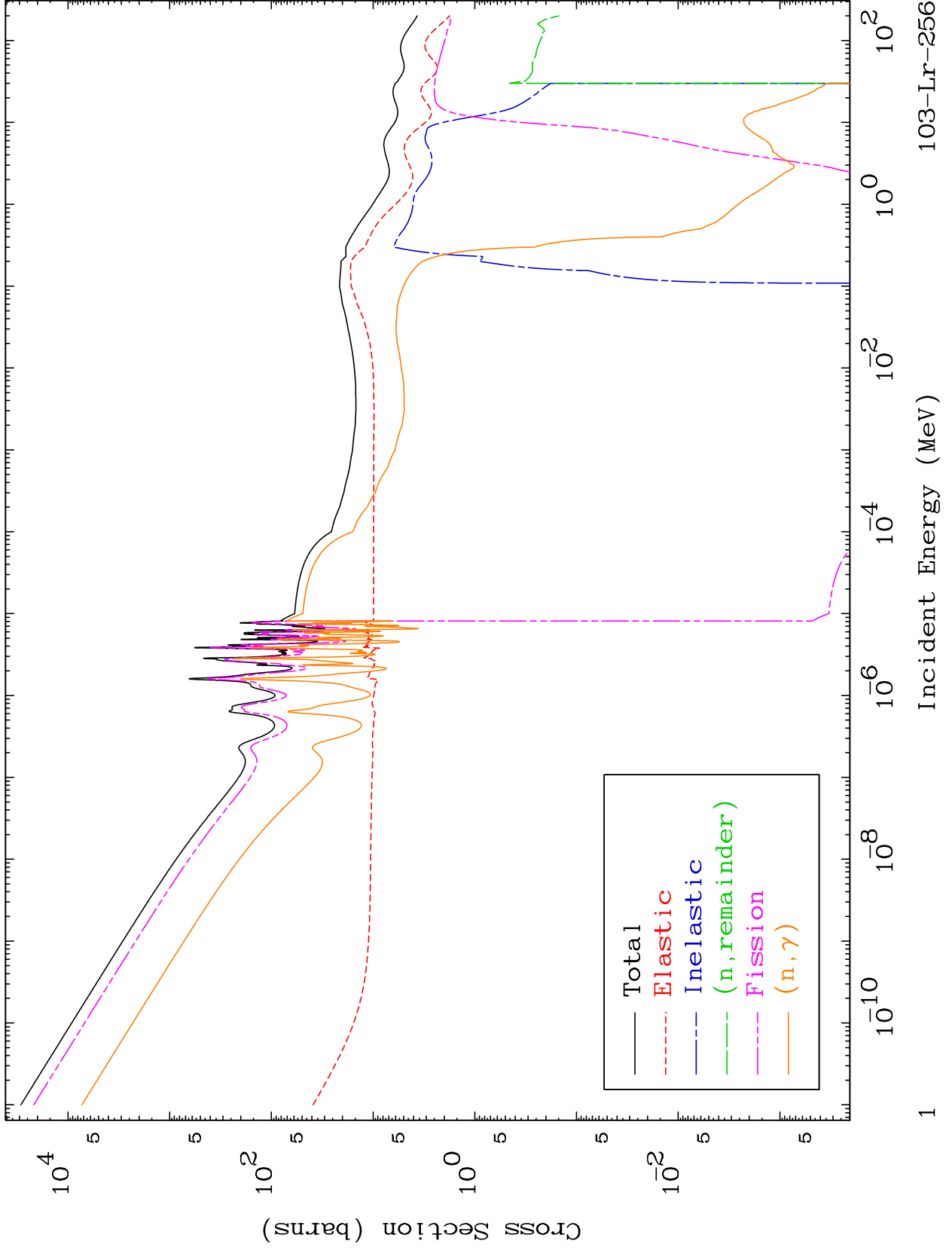
Press Mouse Button to Start

MAT 9985

Major

293 Kelvin Cross Sections

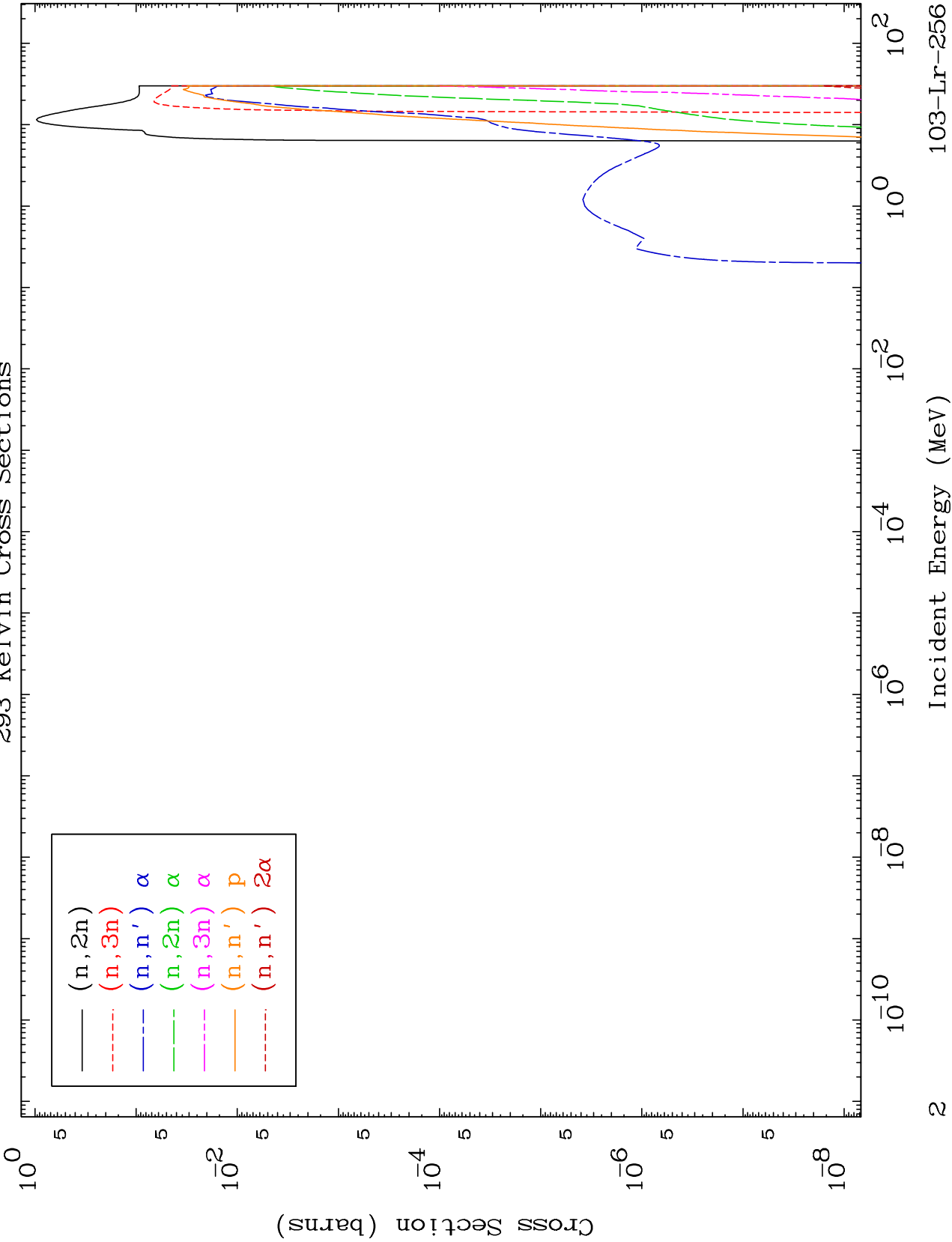
103-Lr-256

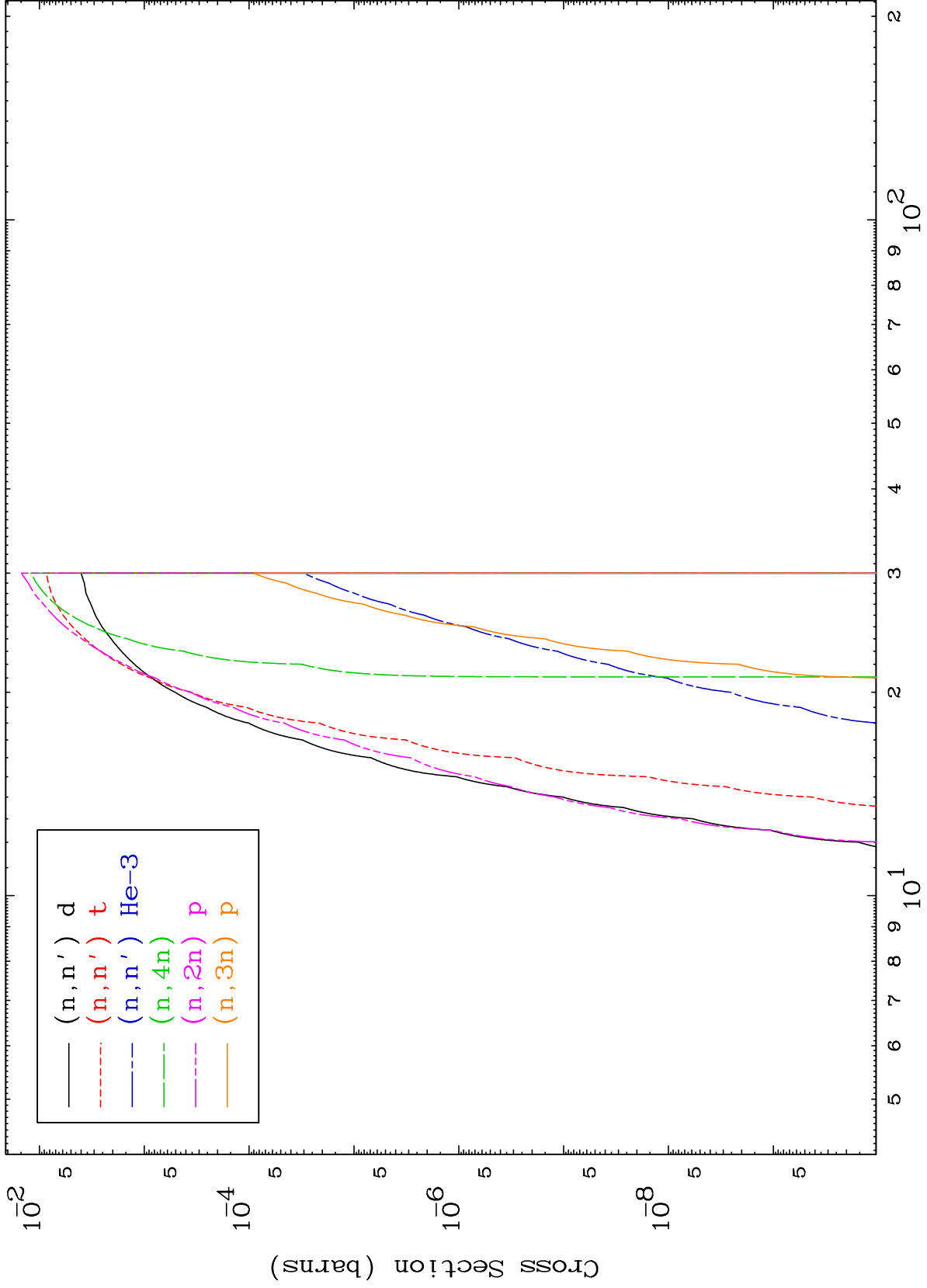


MAT 9985

Neutron Production
293 Kelvin Cross Sections

103-Lr-256

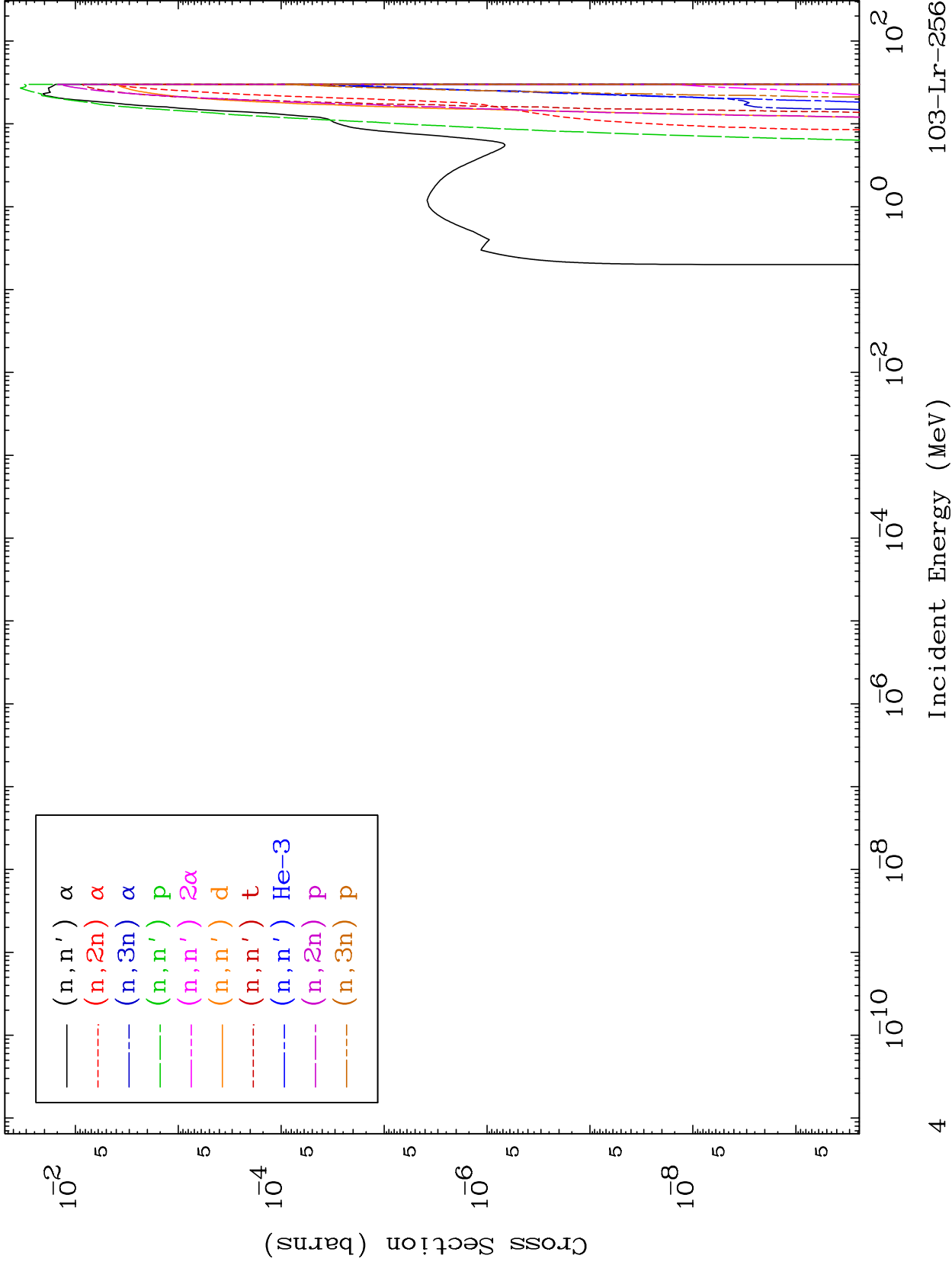




MAT 9985

Charged Particle
293 Kelvin Cross Sections

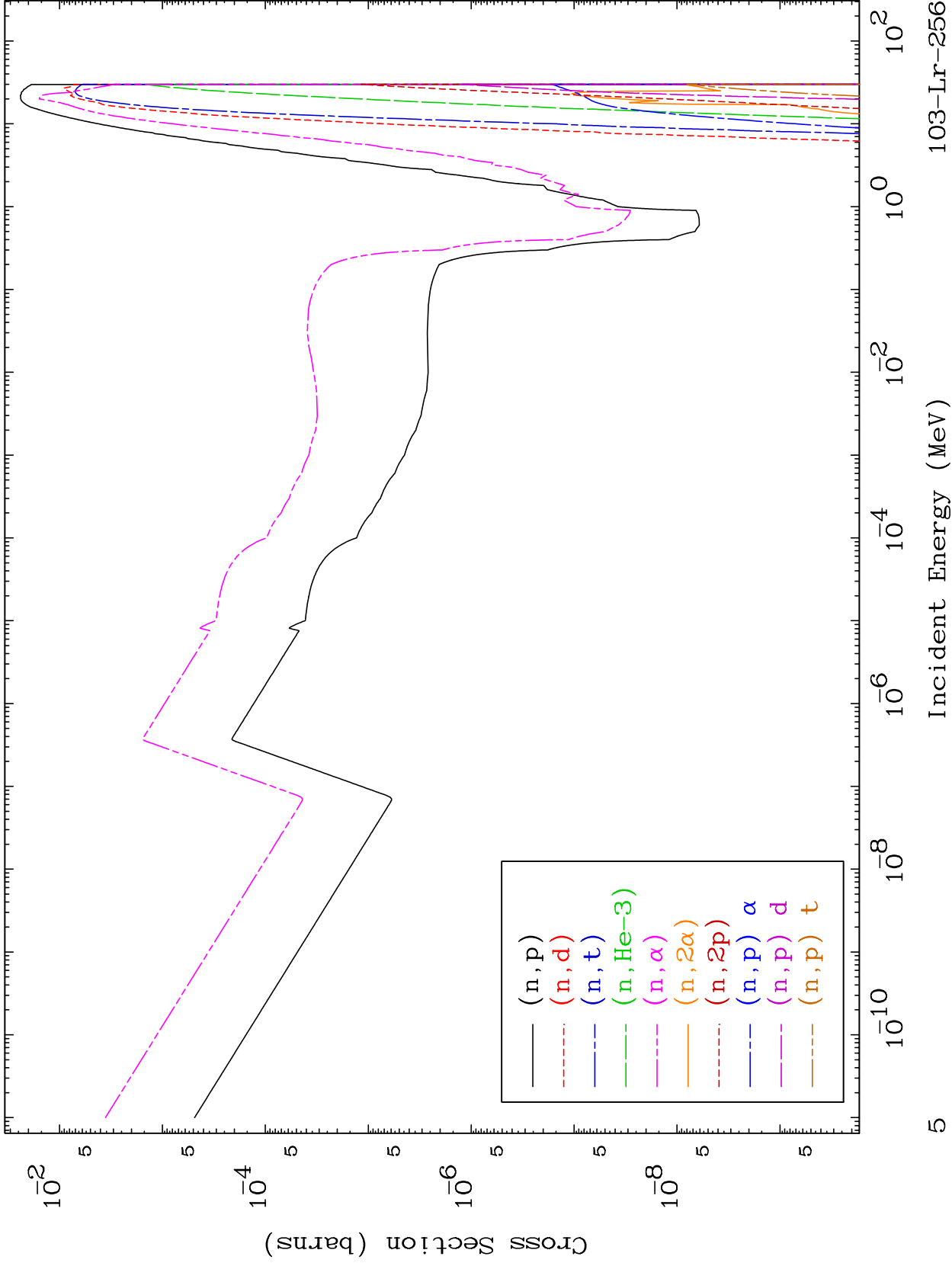
103-Lr-256



MAT 9985

Charged Particle
293 Kelvin Cross Sections

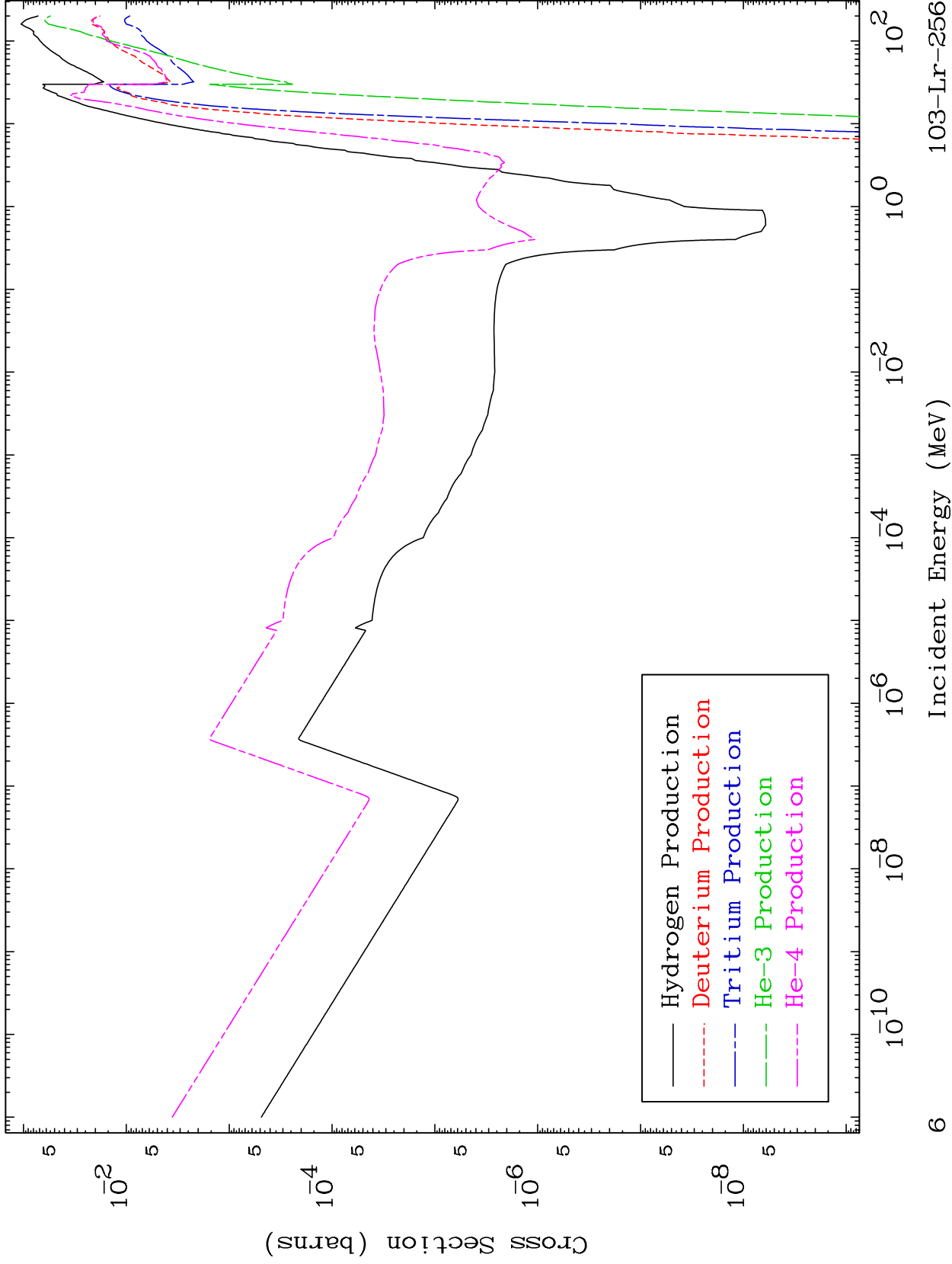
103-Lr-256



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Particle Production
293 Kelvin Cross Sections

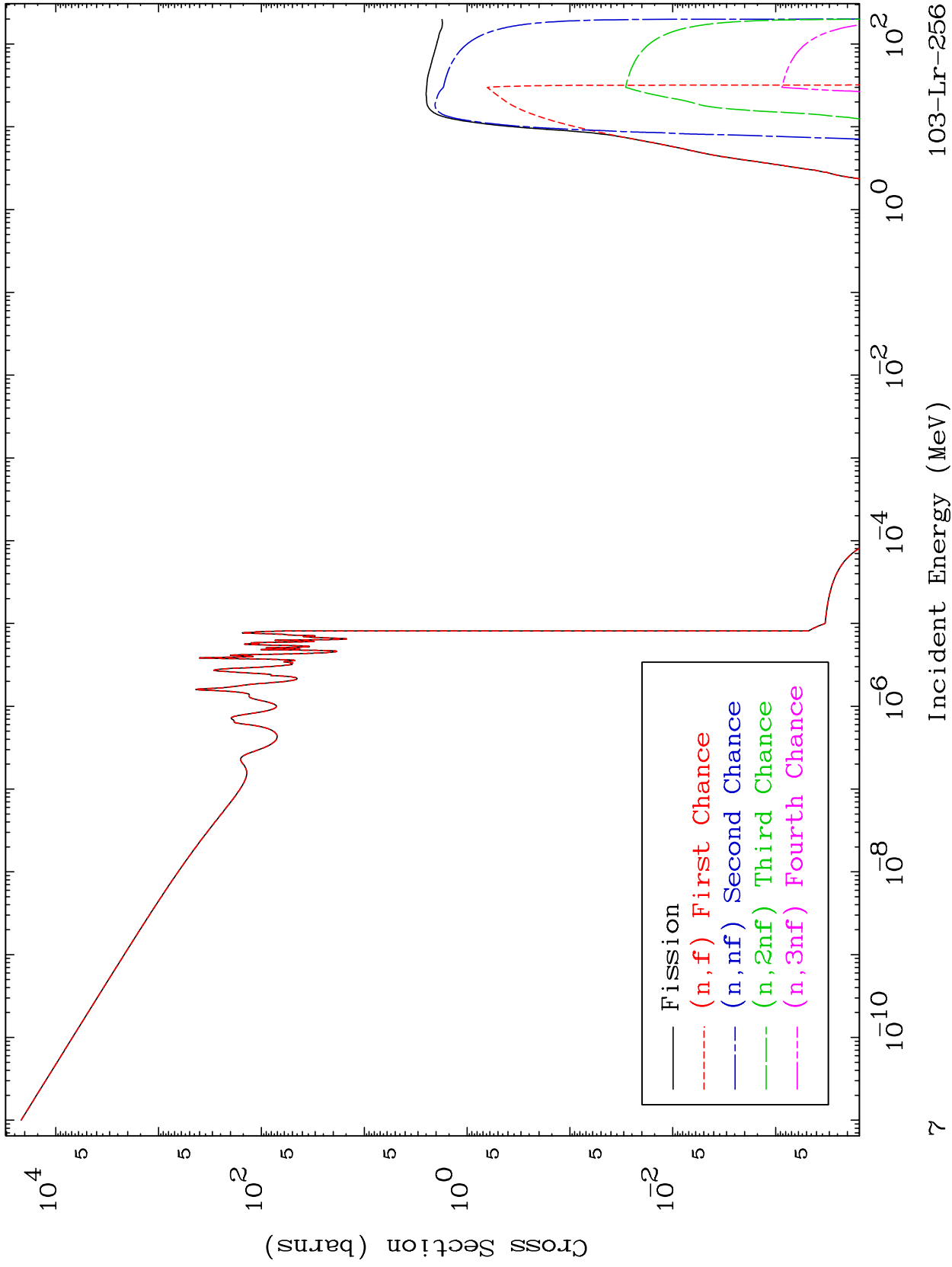
103-Lr-256



MAT 9985

Fission
293 Kelvin Cross Sections

103-Lr-256

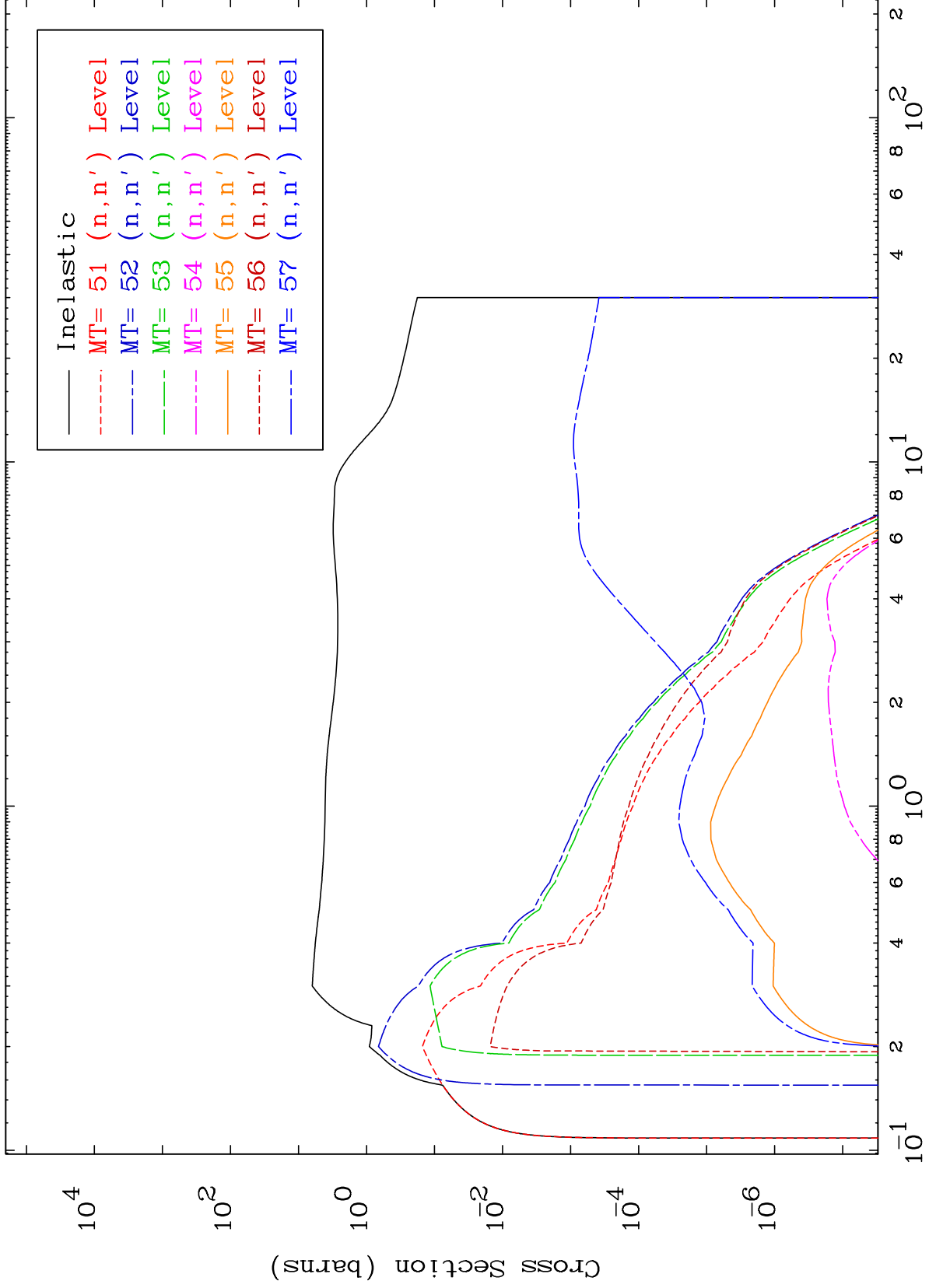


MAT 9985

(n,n') Level

103-Lr-256

293 Kelvin Cross Sections



Incident Energy (MeV)

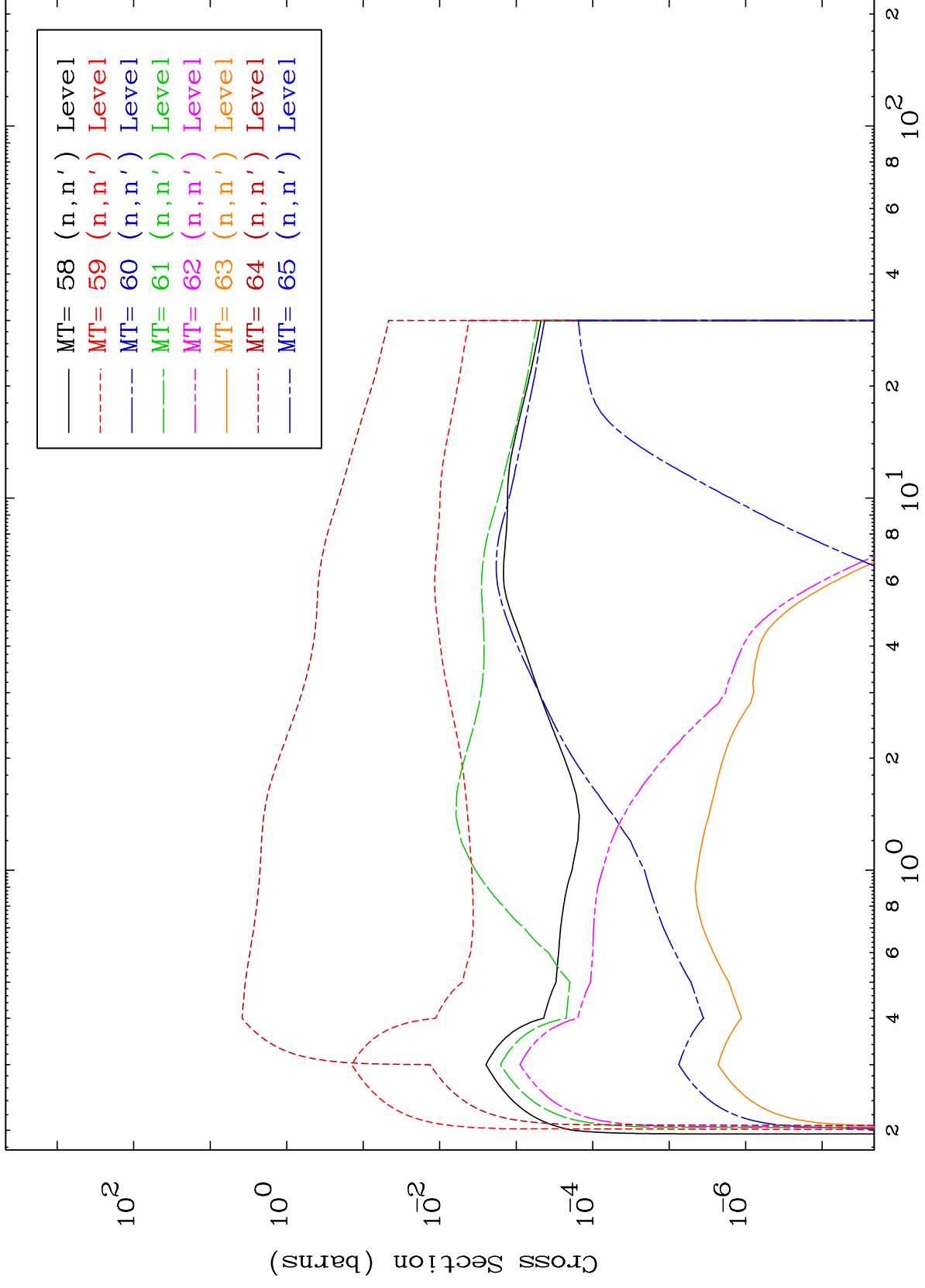
103-Lr-256

MAT 9985

(n,n') Level

103-Lr-256

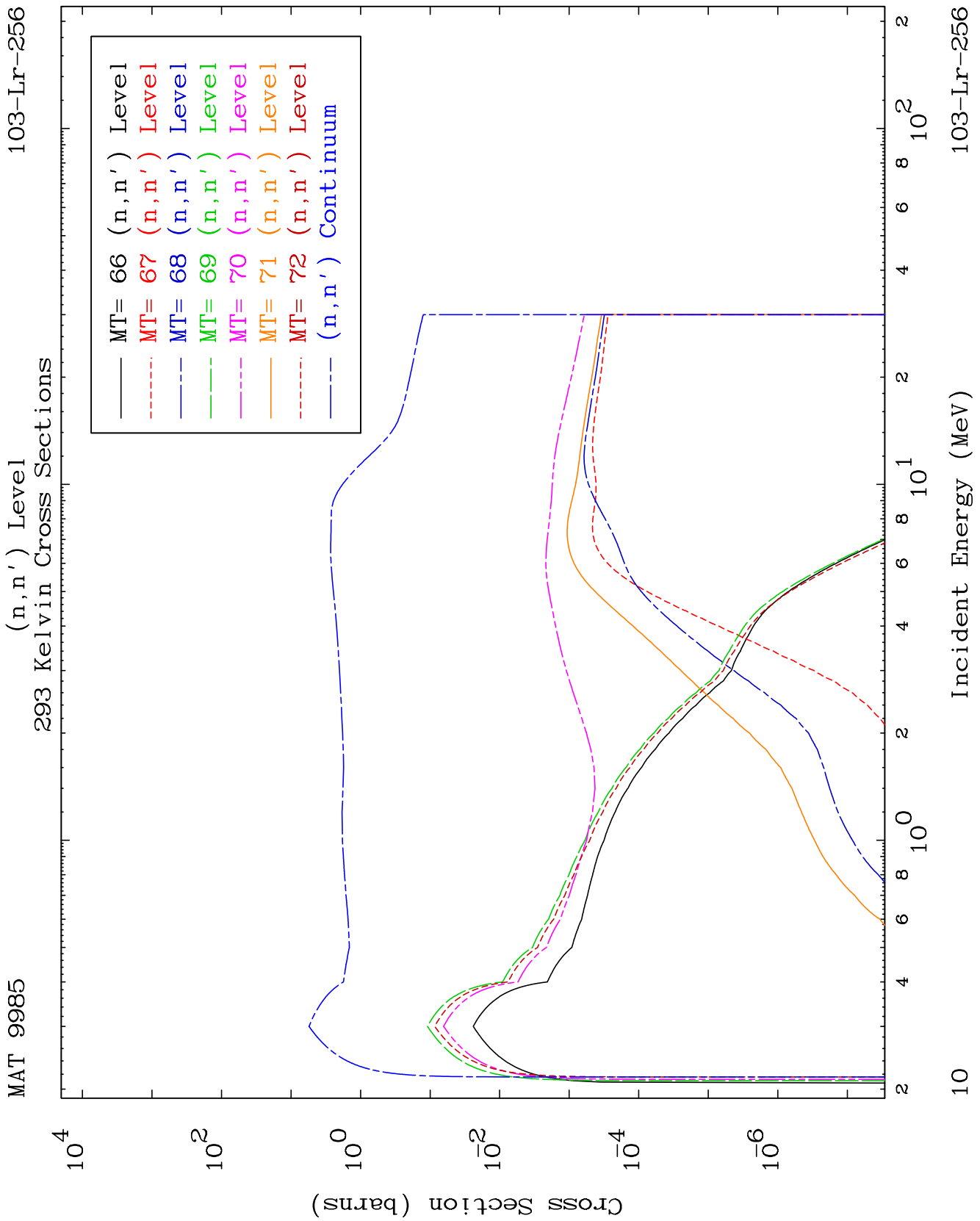
293 Kelvin Cross Sections



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Incident Energy (MeV)

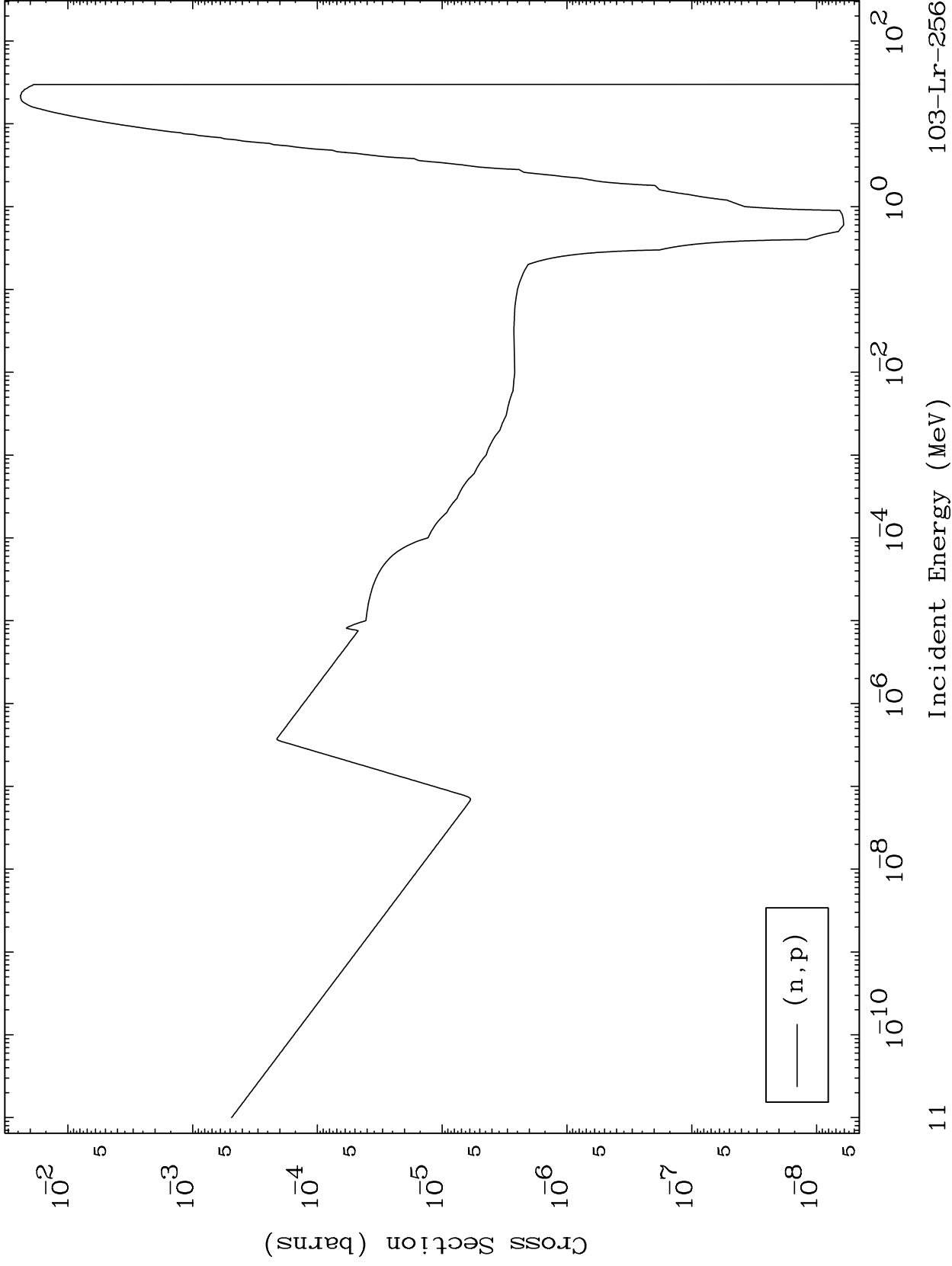
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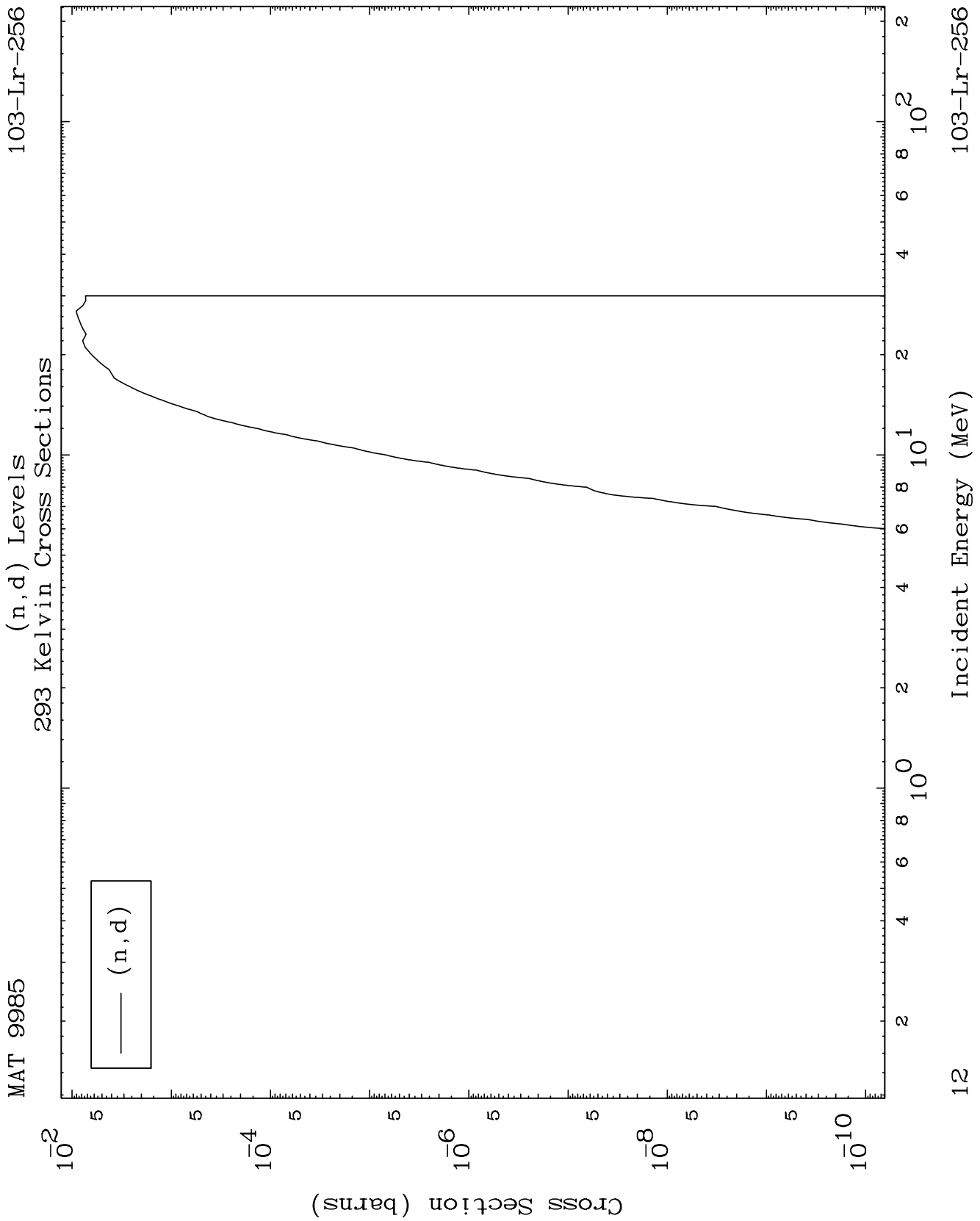


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(n,p) Levels
293 Kelvin Cross Sections

103-Lr-256

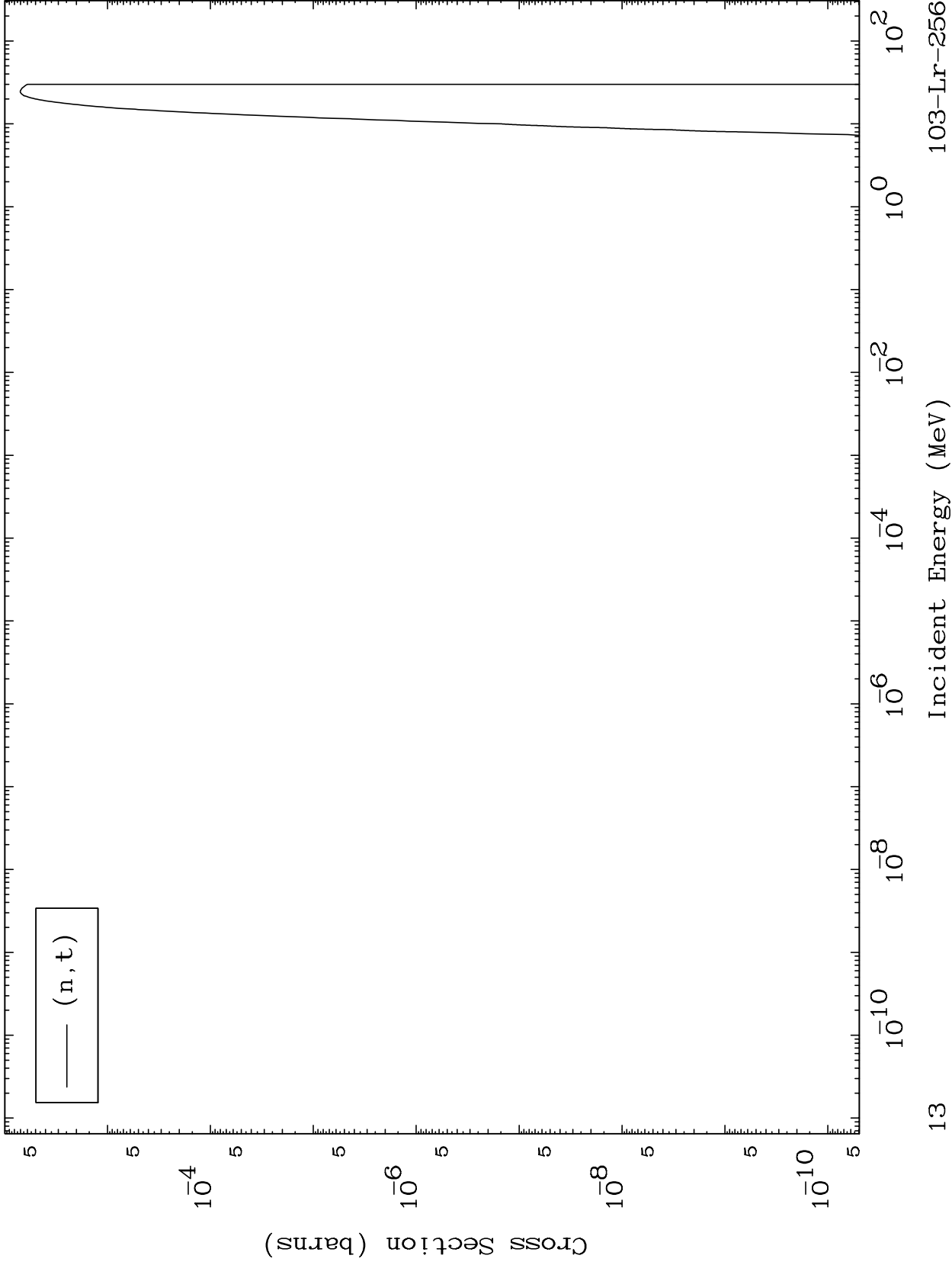




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(n,t) Levels
293 Kelvin Cross Sections

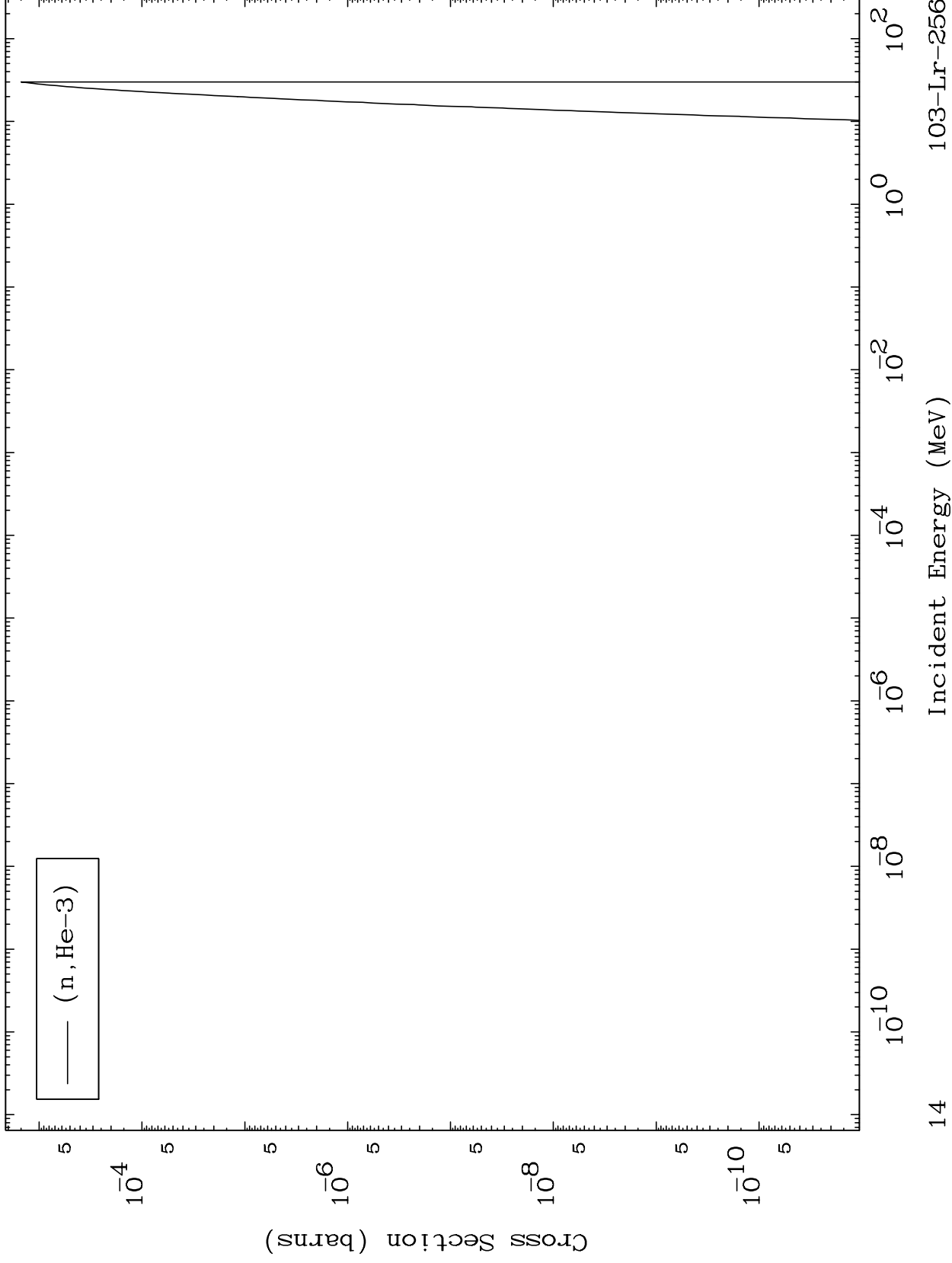
103-Lr-256



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(n,He3) Levels
293 Kelvin Cross Sections

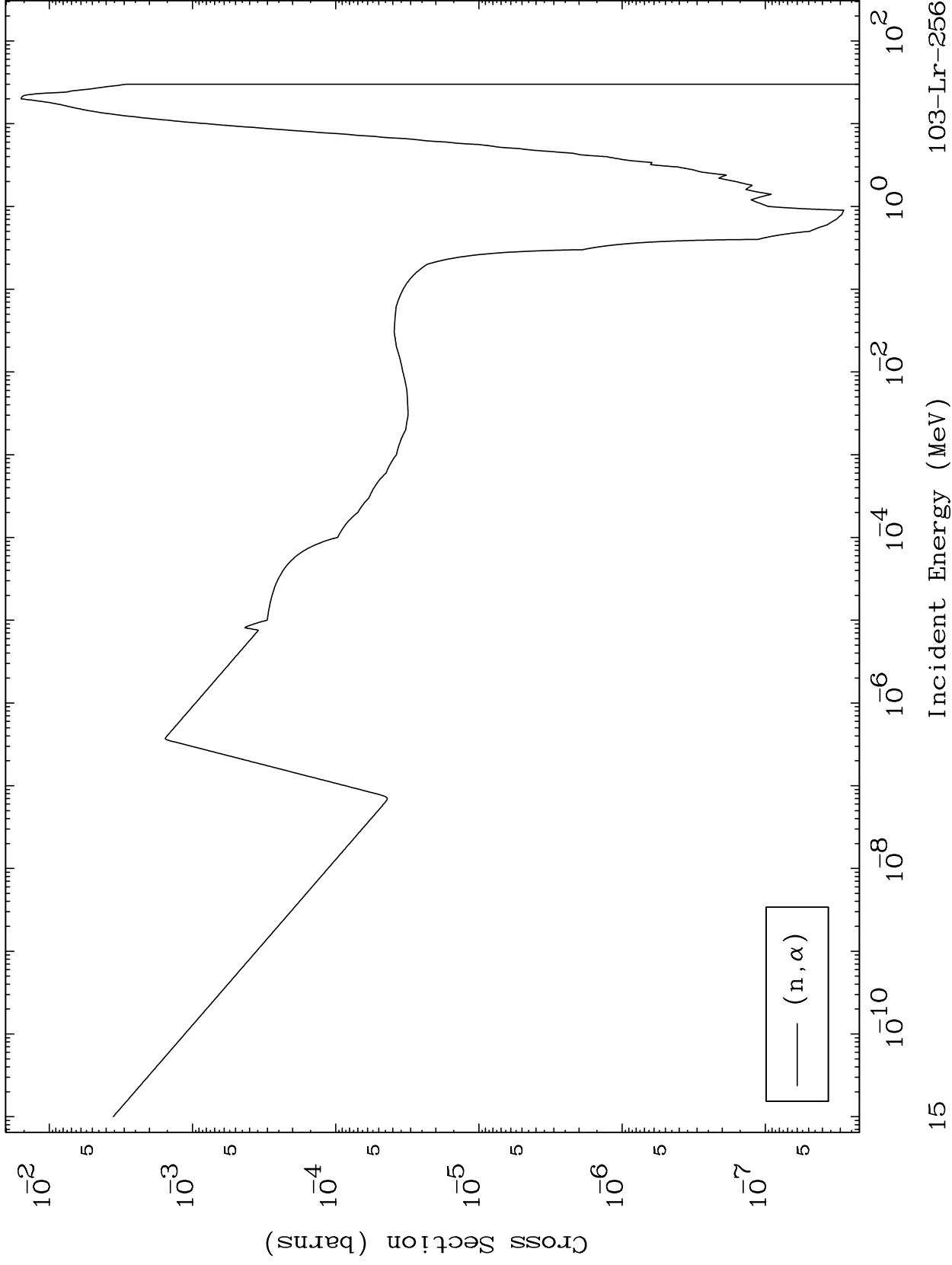
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MAT 9985

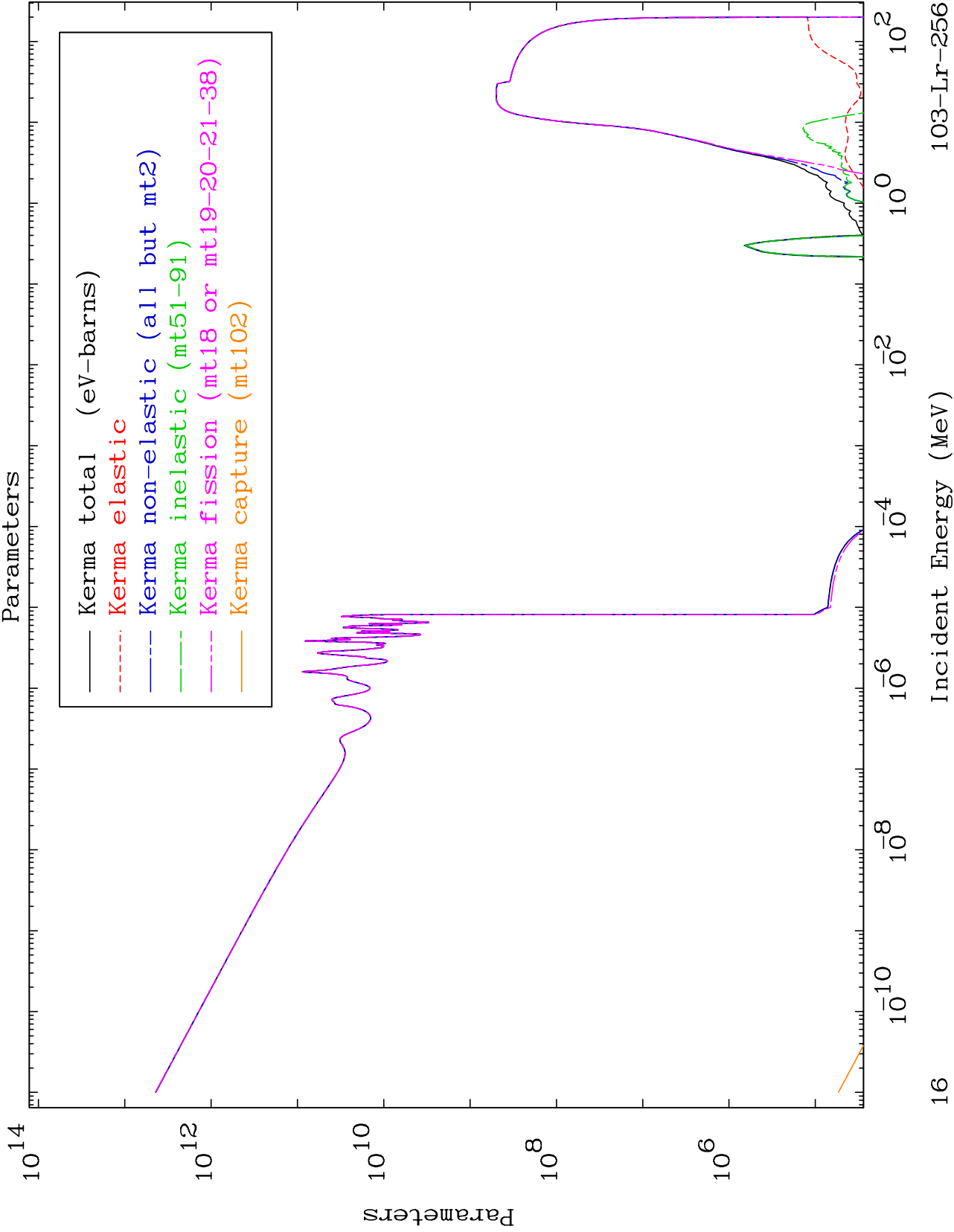
(n,α) Levels
293 Kelvin Cross Sections

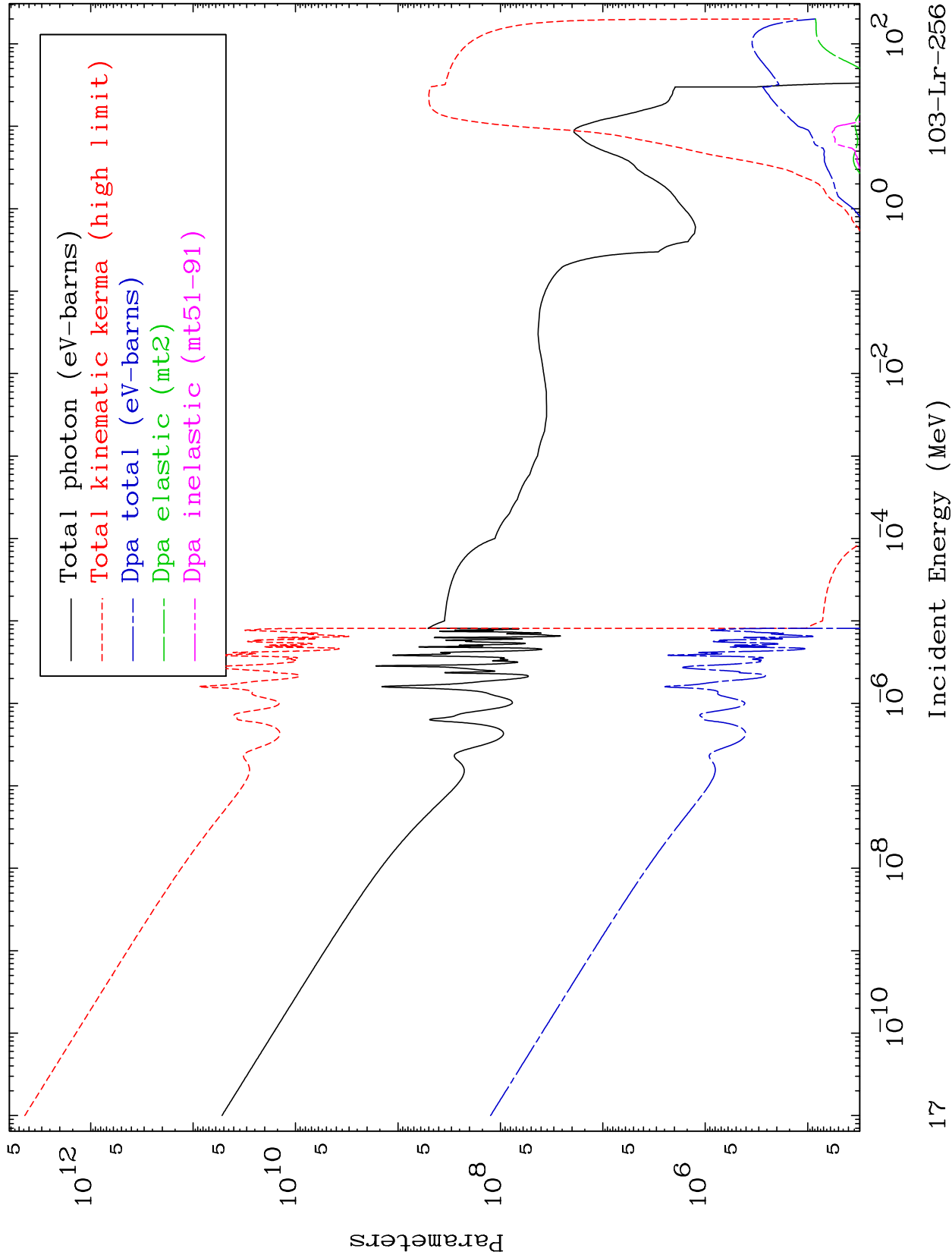
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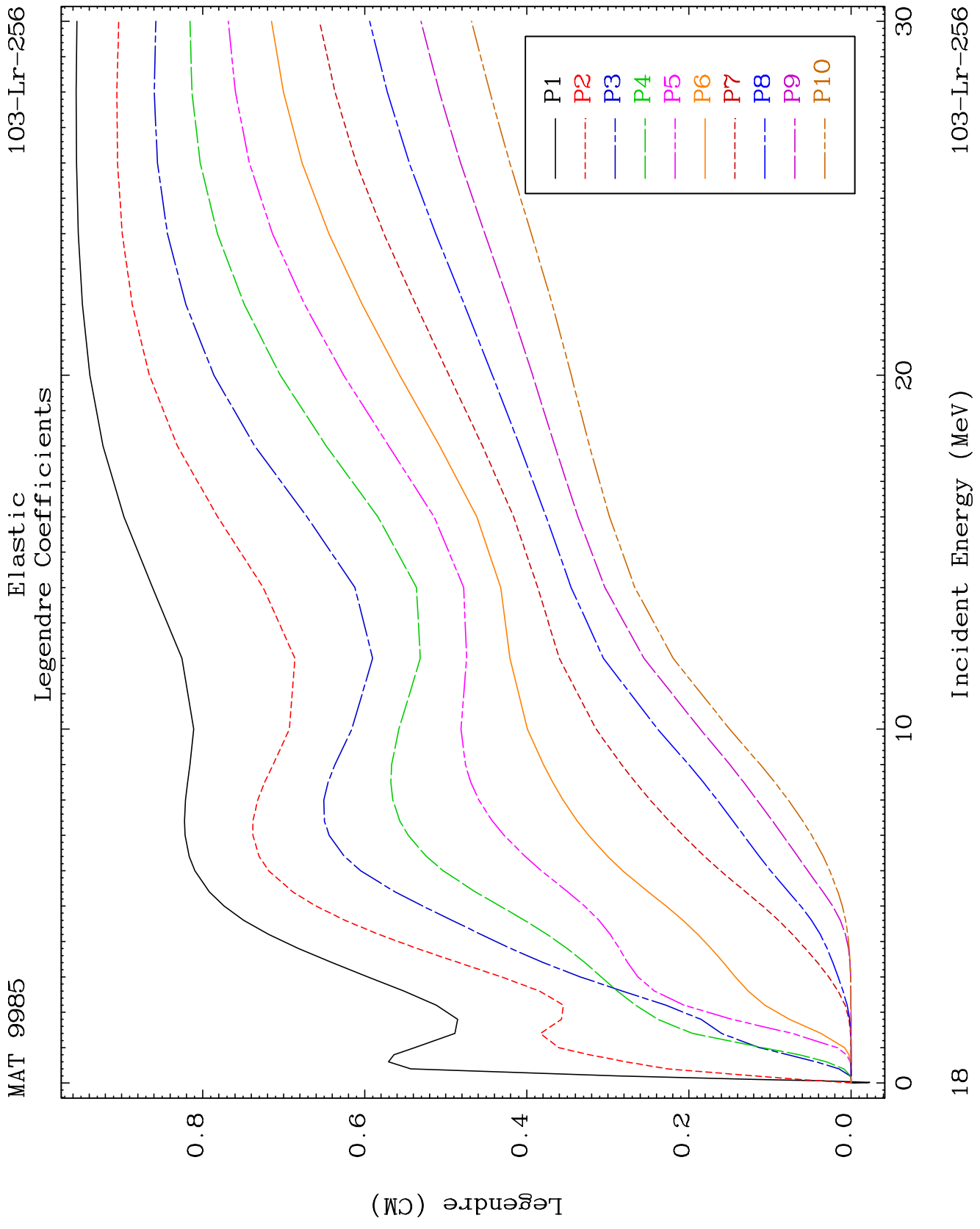


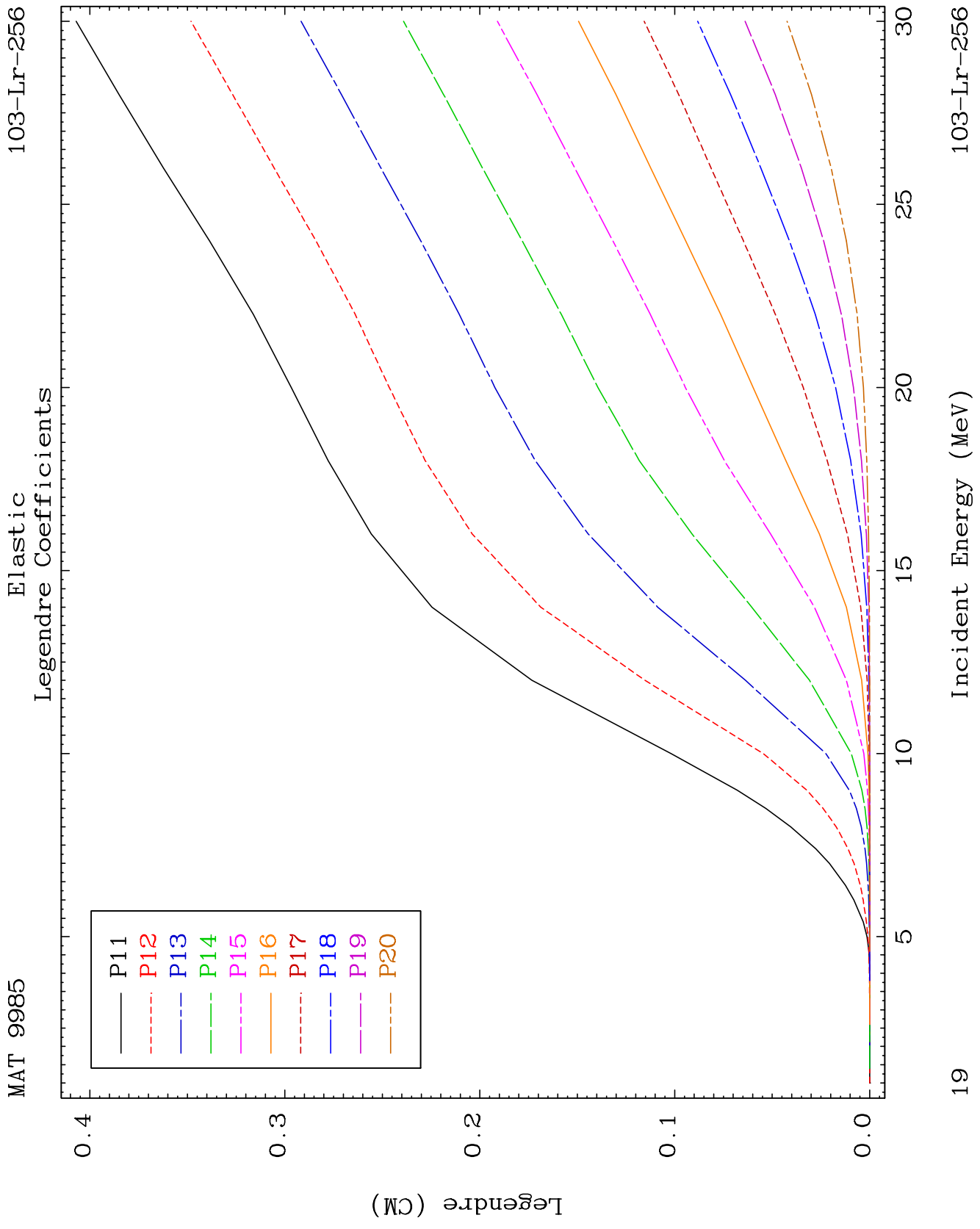
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103-Lr-256





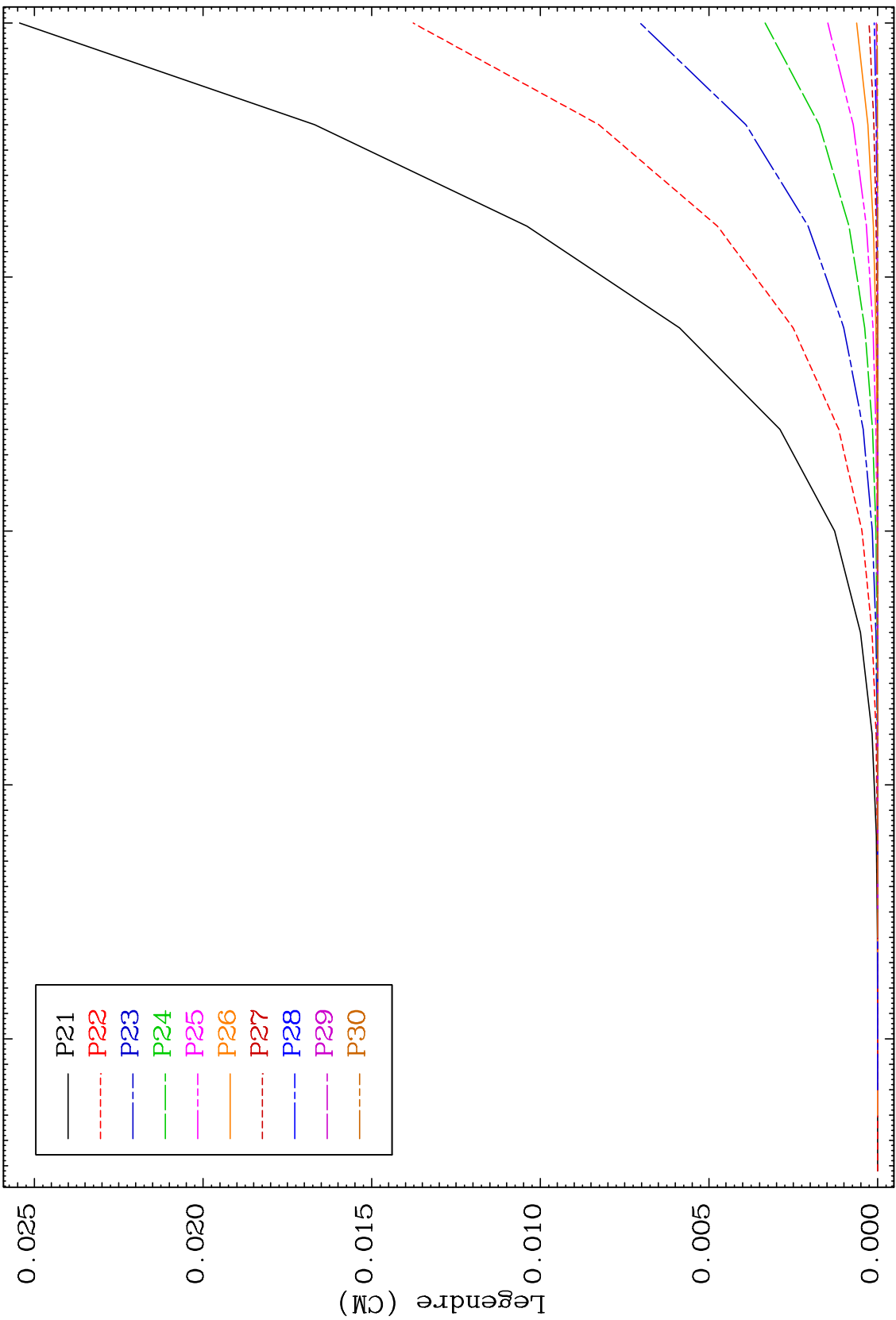




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Elastic
Legendre Coefficients

103-Lr-256



20

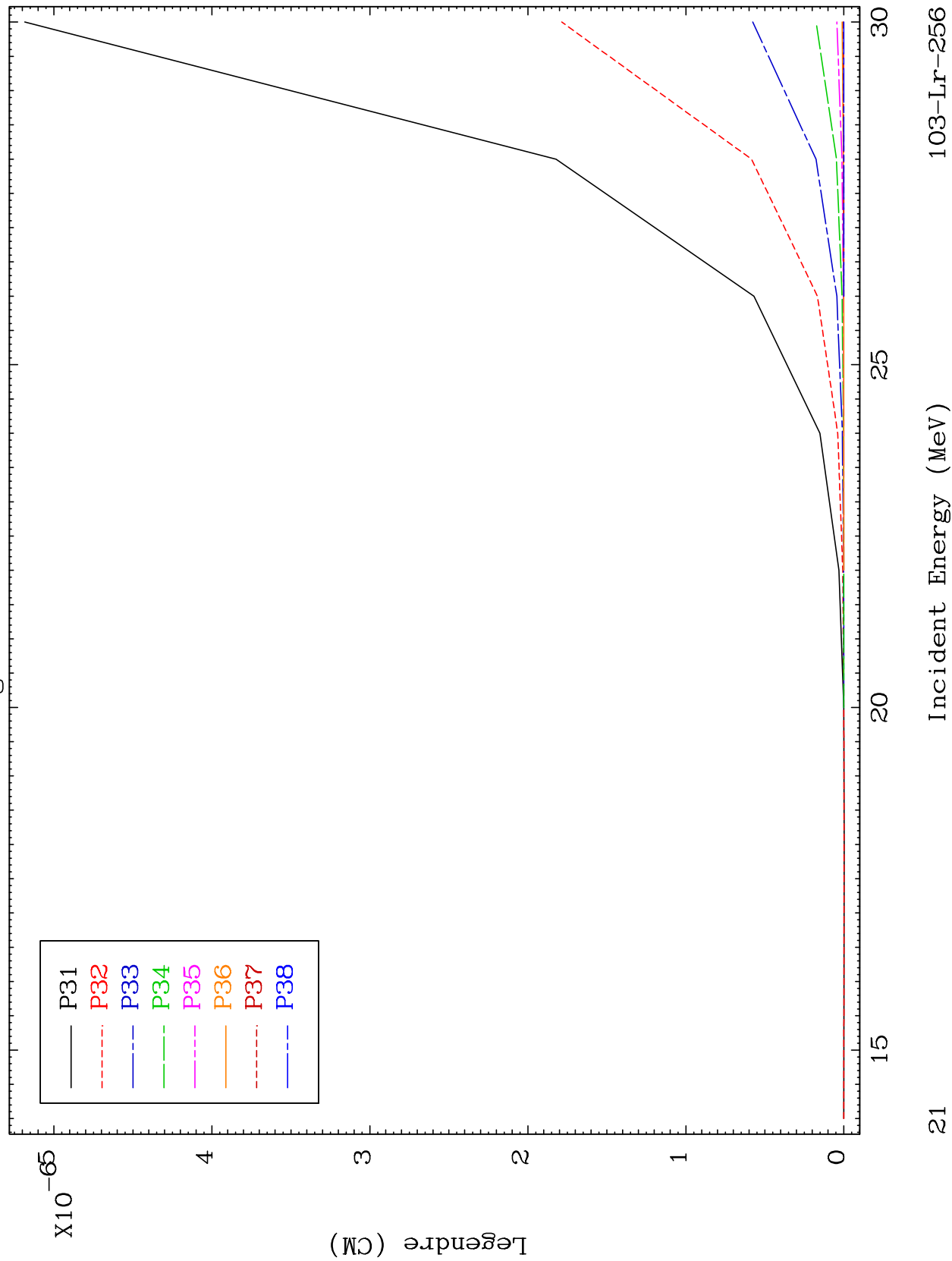
Incident Energy (MeV)

103-Lr-256

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Elastic Legendre Coefficients

103-Lr-256



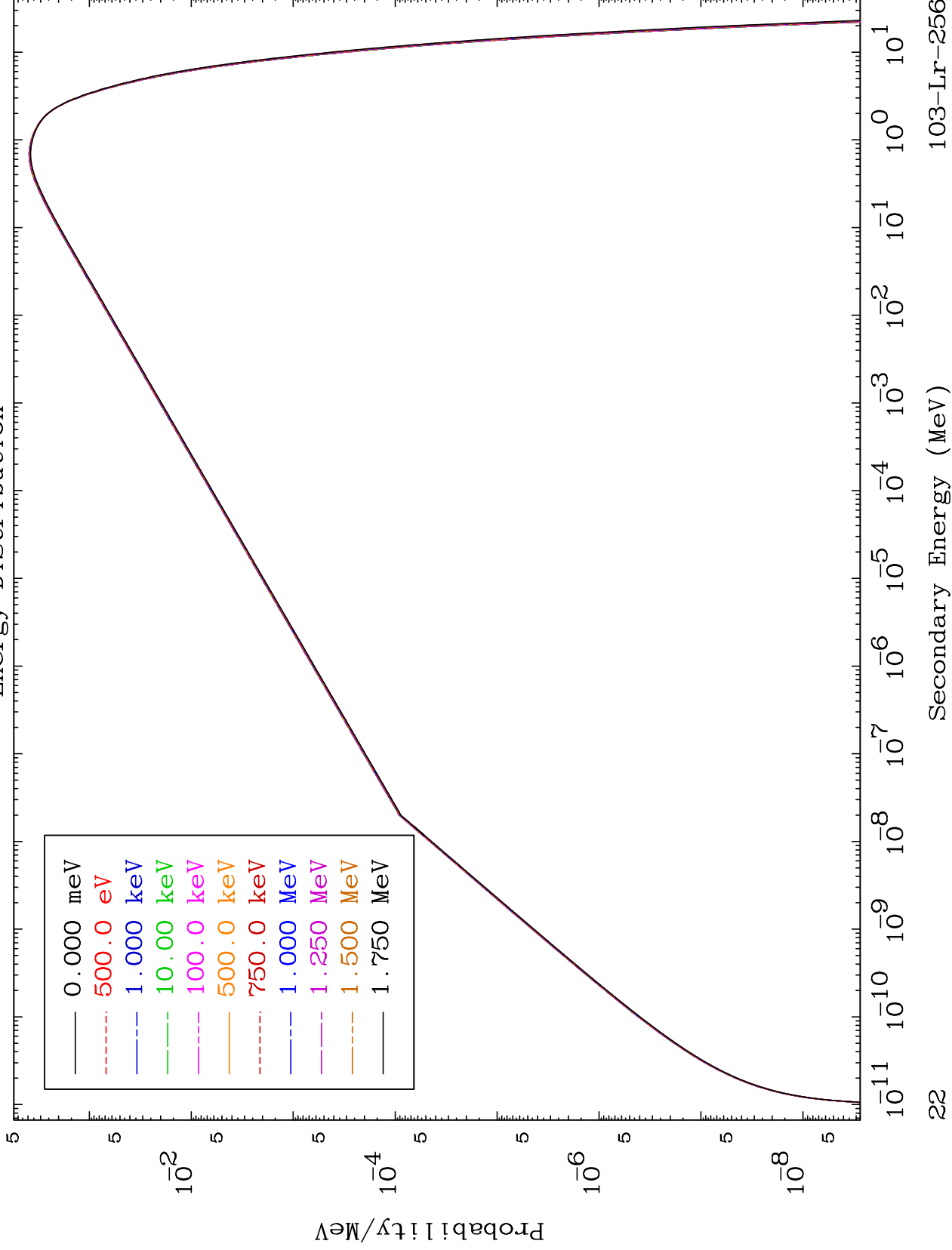
21

103-Lr-256

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Fission
Energy Distribution

103-Lr-256



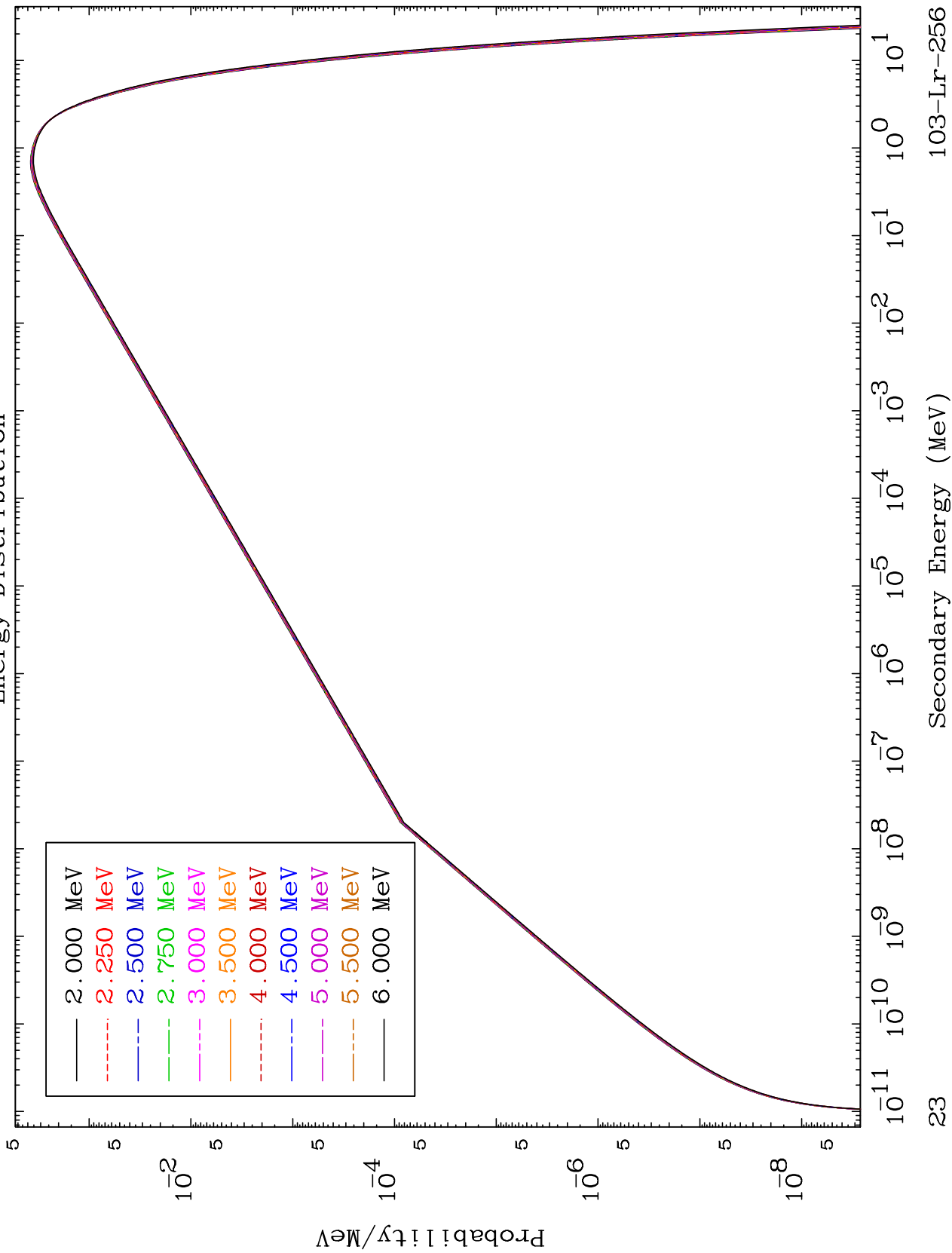
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103-Lr-256

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Fission Energy Distribution

103-Lr-256



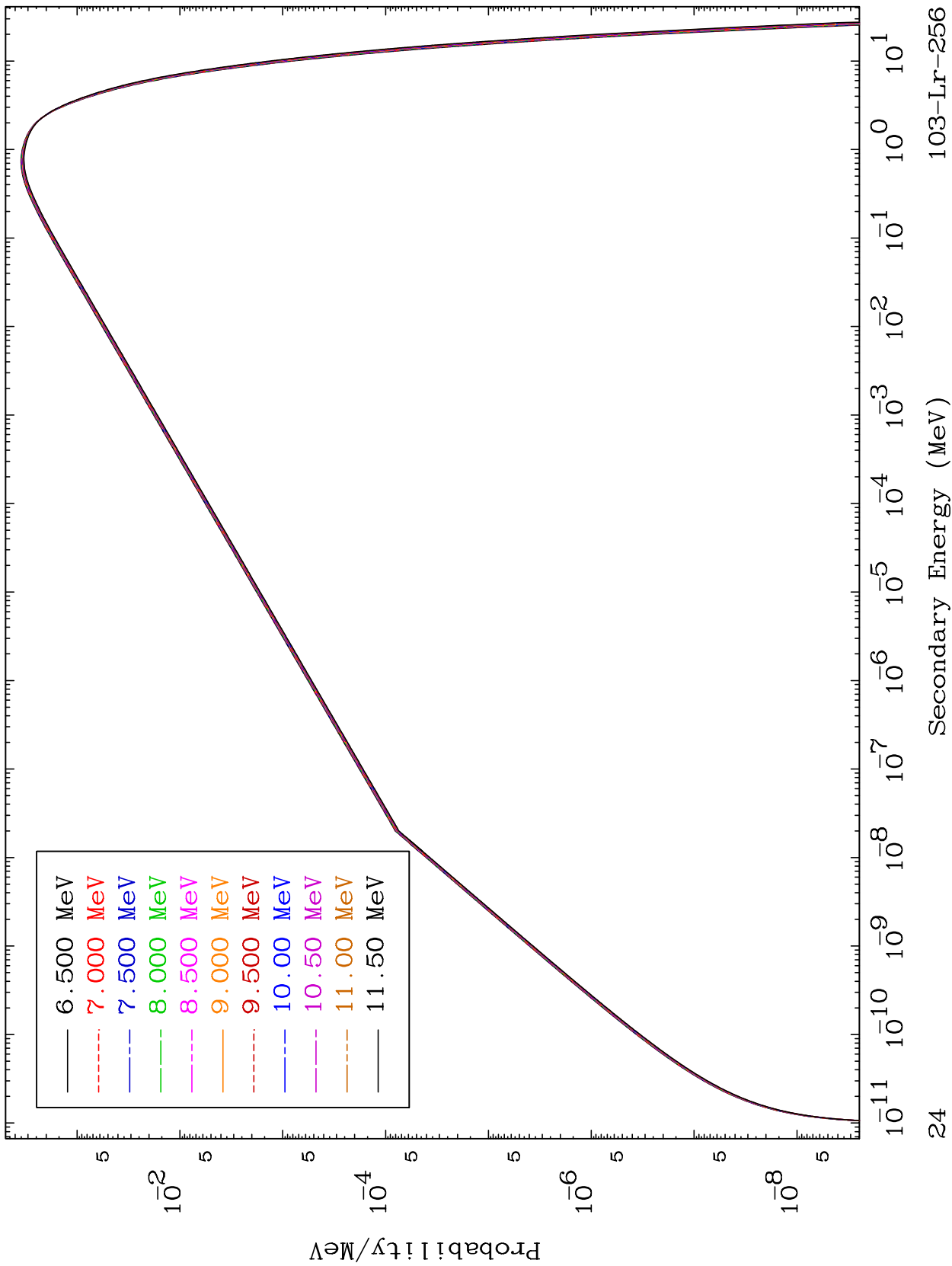
103-Lr-256

Secondary Energy (MeV)

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Fission Energy Distribution

103-Lr-256



103-Lr-256

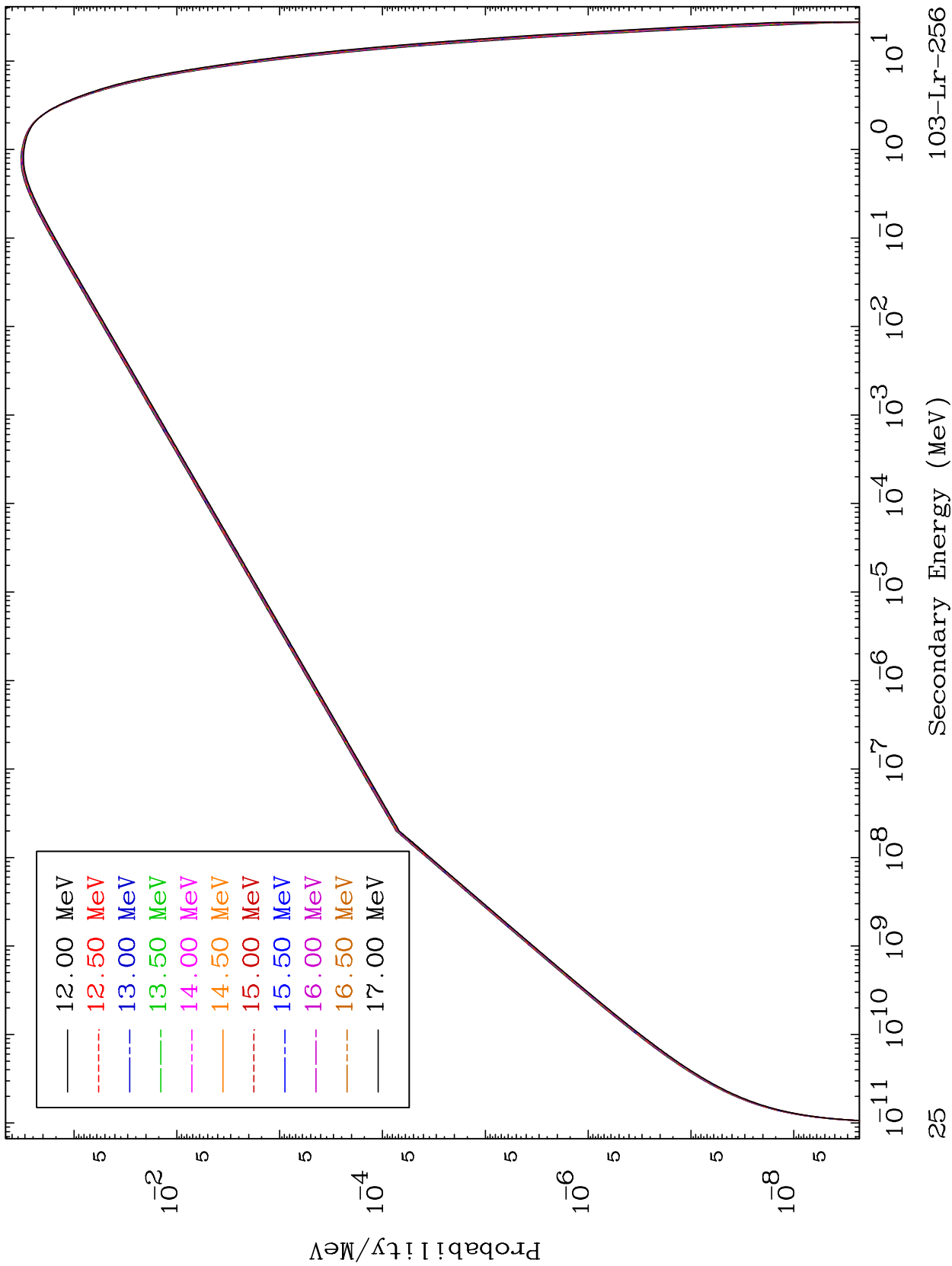
Secondary Energy (MeV)

24

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Fission
Energy Distribution

103-Lr-256



103-Lr-256

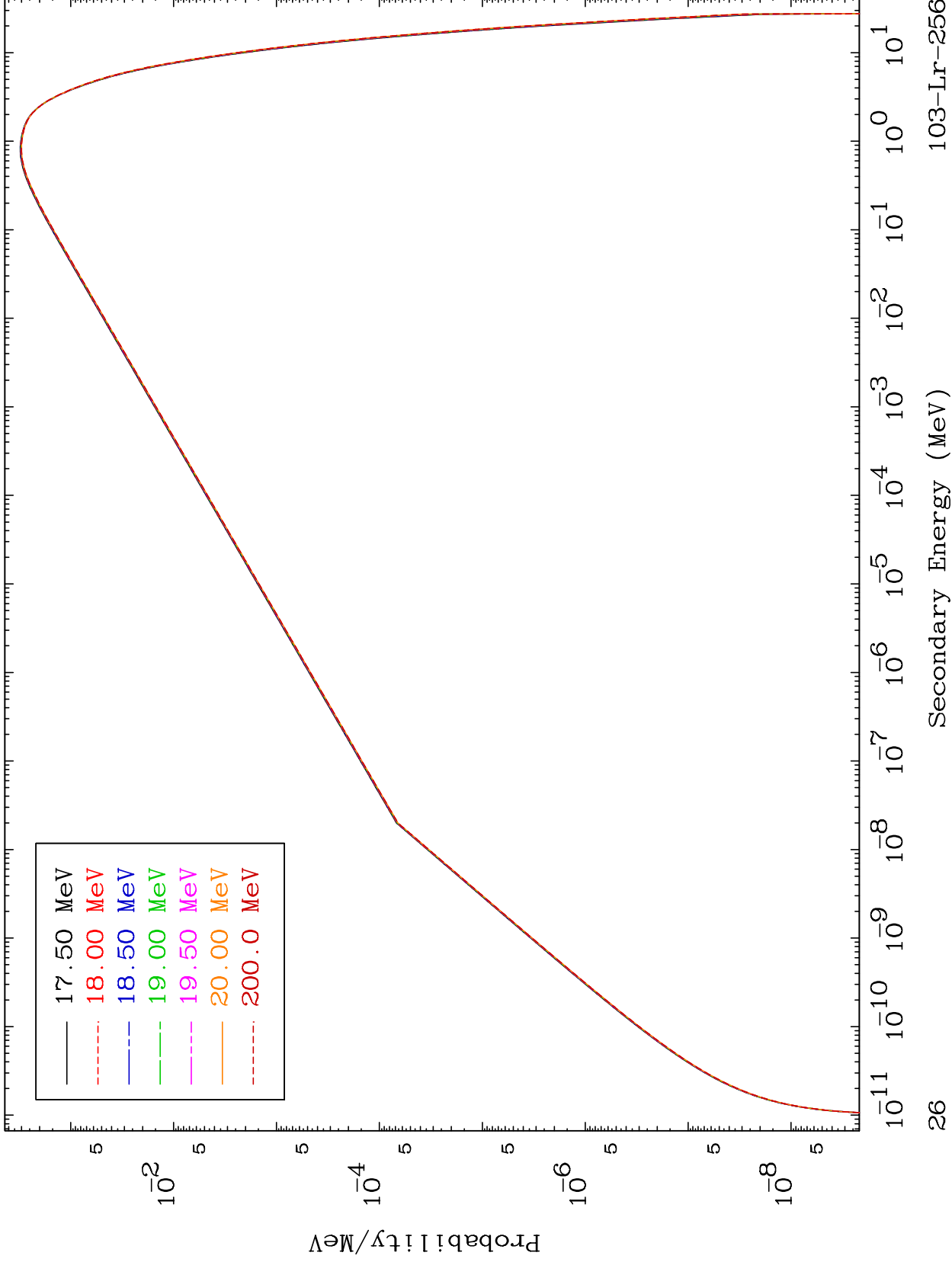
Secondary Energy (MeV)

25

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Fission
Energy Distribution

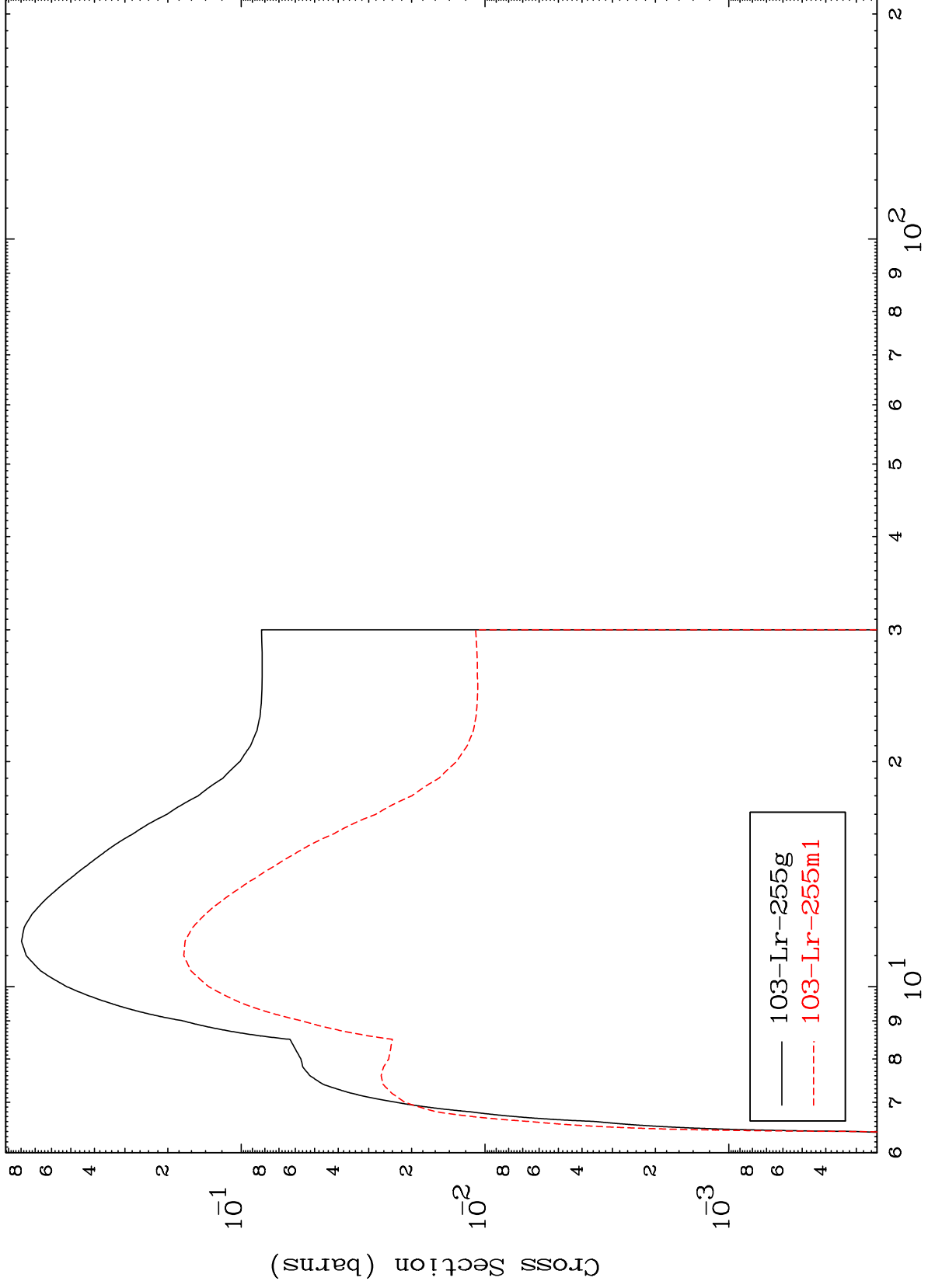
103-Lr-256



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103-Lr-256

(n,2n)
Radionuclide Production Cross Section



27

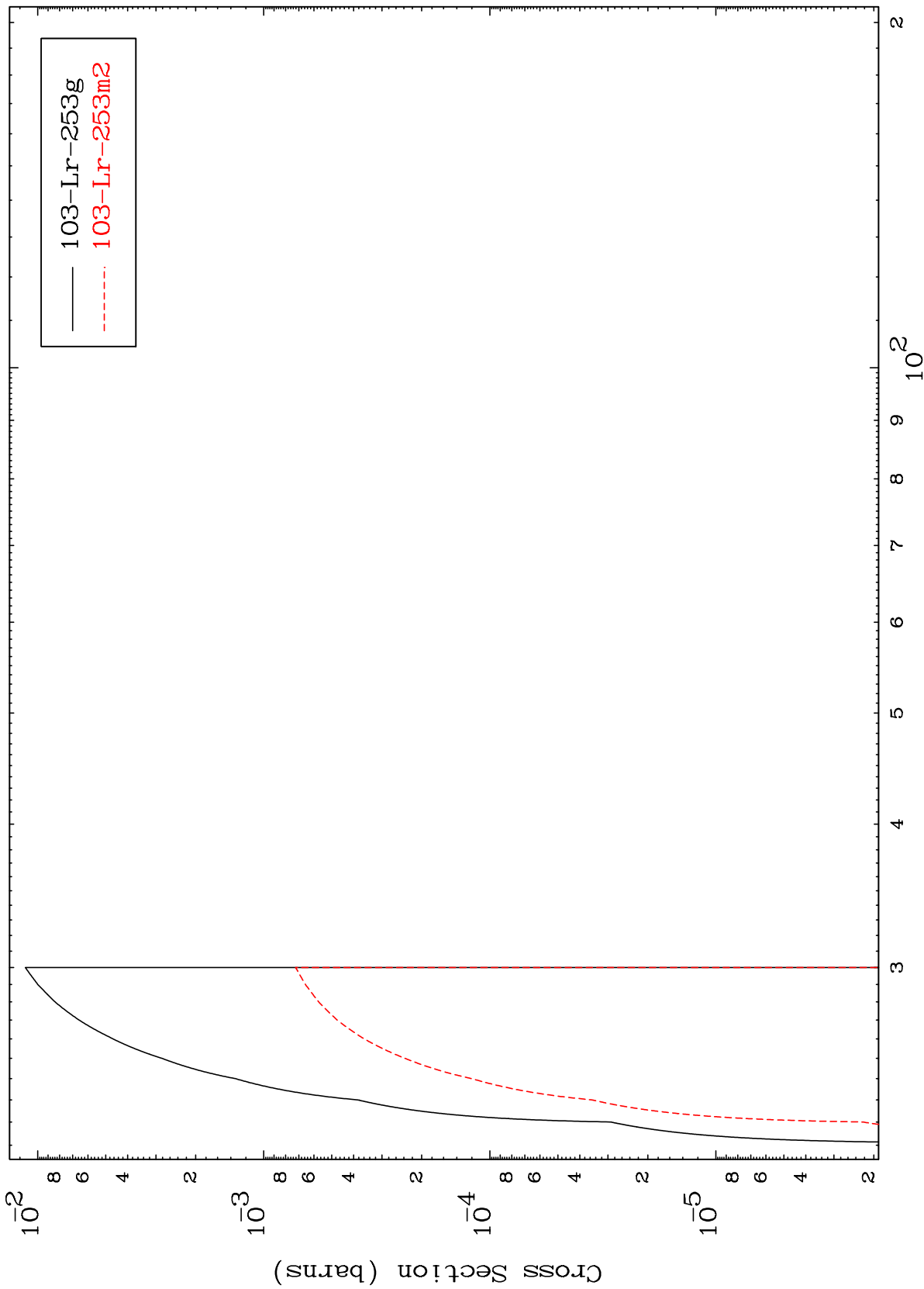
Incident Energy (MeV)

103-Lr-256

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103-Lr-256

(n,4n)
Radionuclide Production Cross Section



28

103-Lr-256

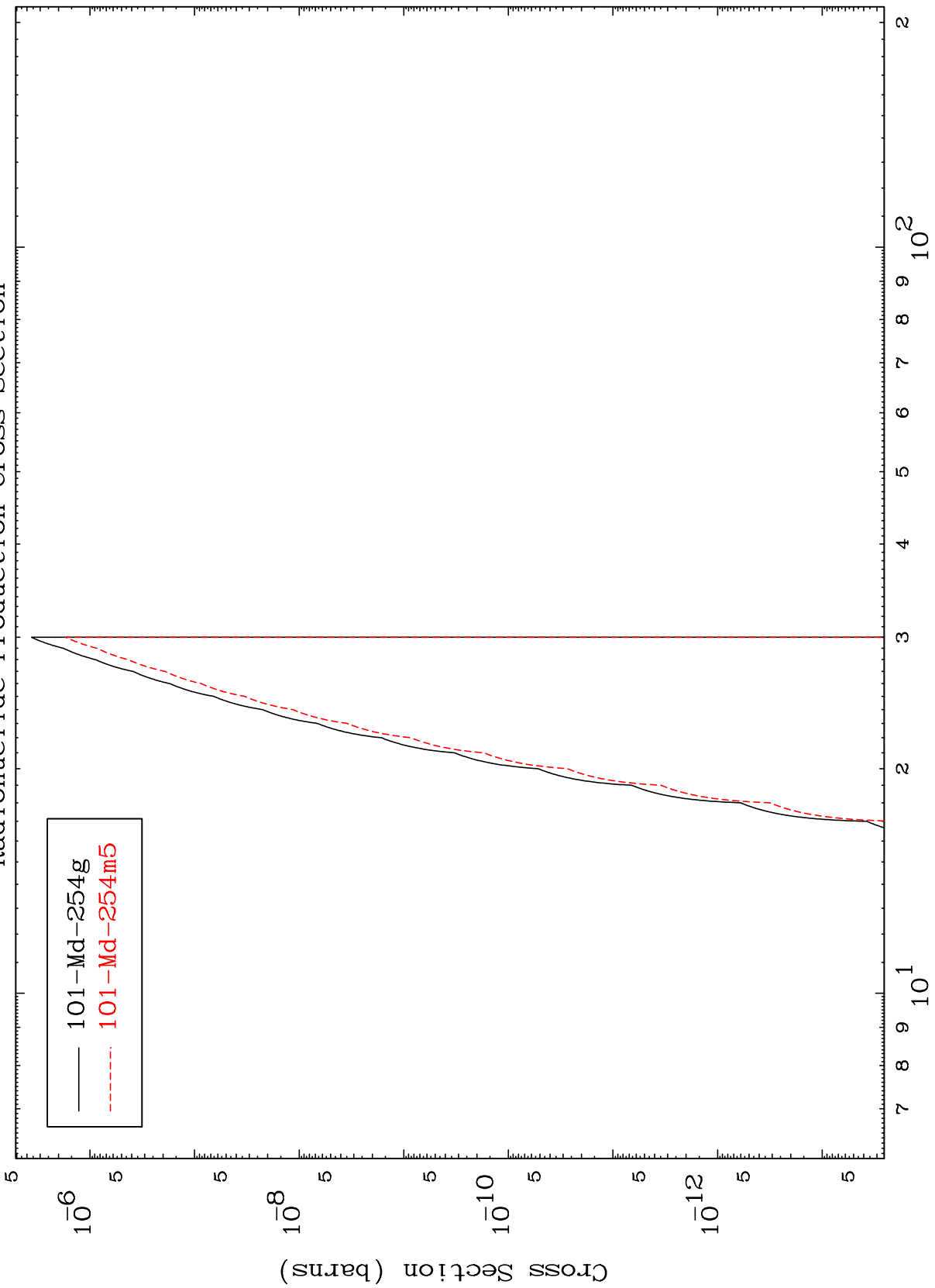
Incident Energy (MeV)

MAT 9985

(n,2n) p

103-Lr-256

Radionuclide Production Cross Section



— 101-Md-254g
- - - 101-Md-254m5

29

Incident Energy (MeV)

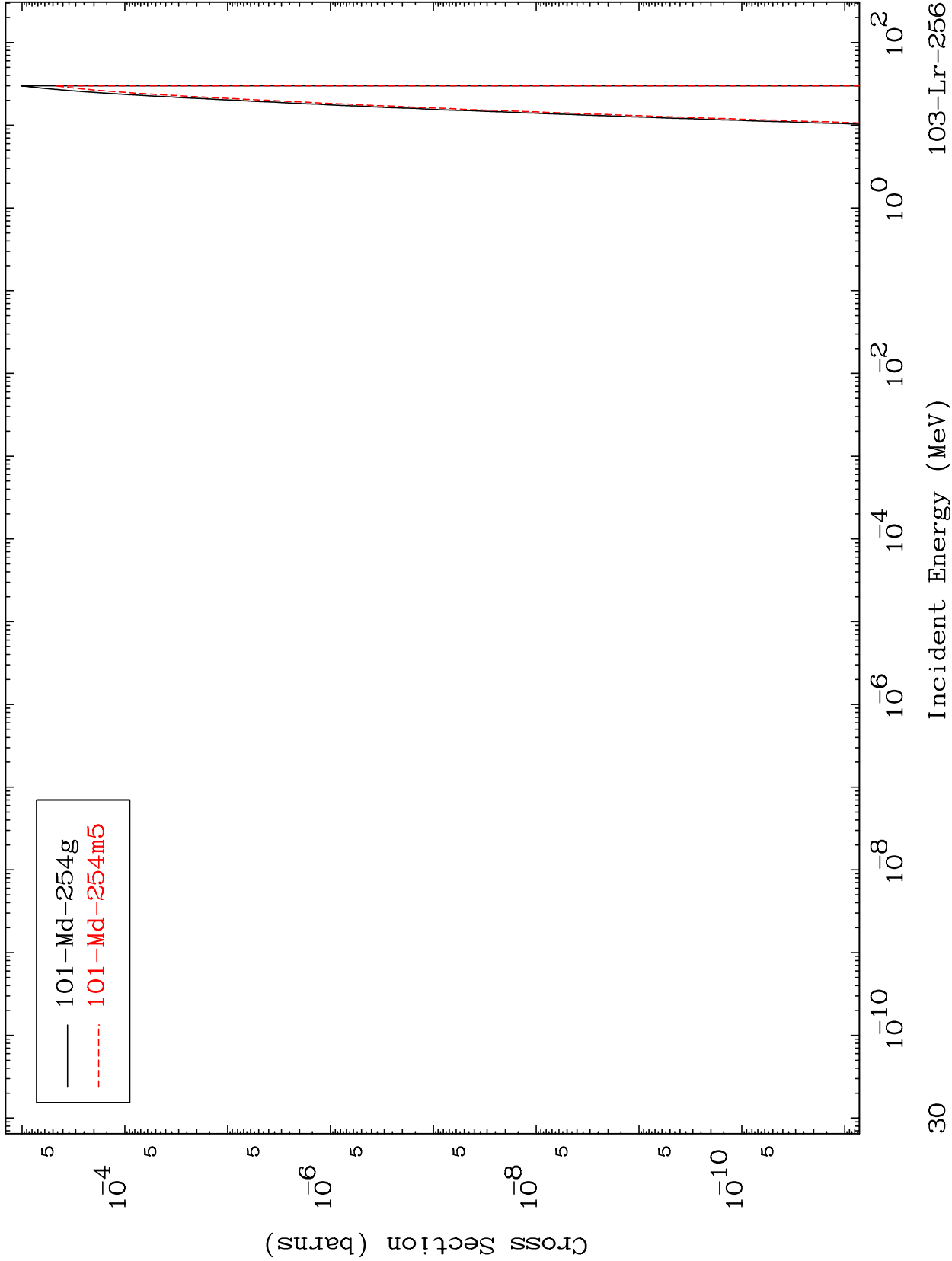
103-Lr-256

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(n,He-3)

103-Lr-256

Radionuclide Production Cross Section



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(n,p) d

103-Lr-256

