

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

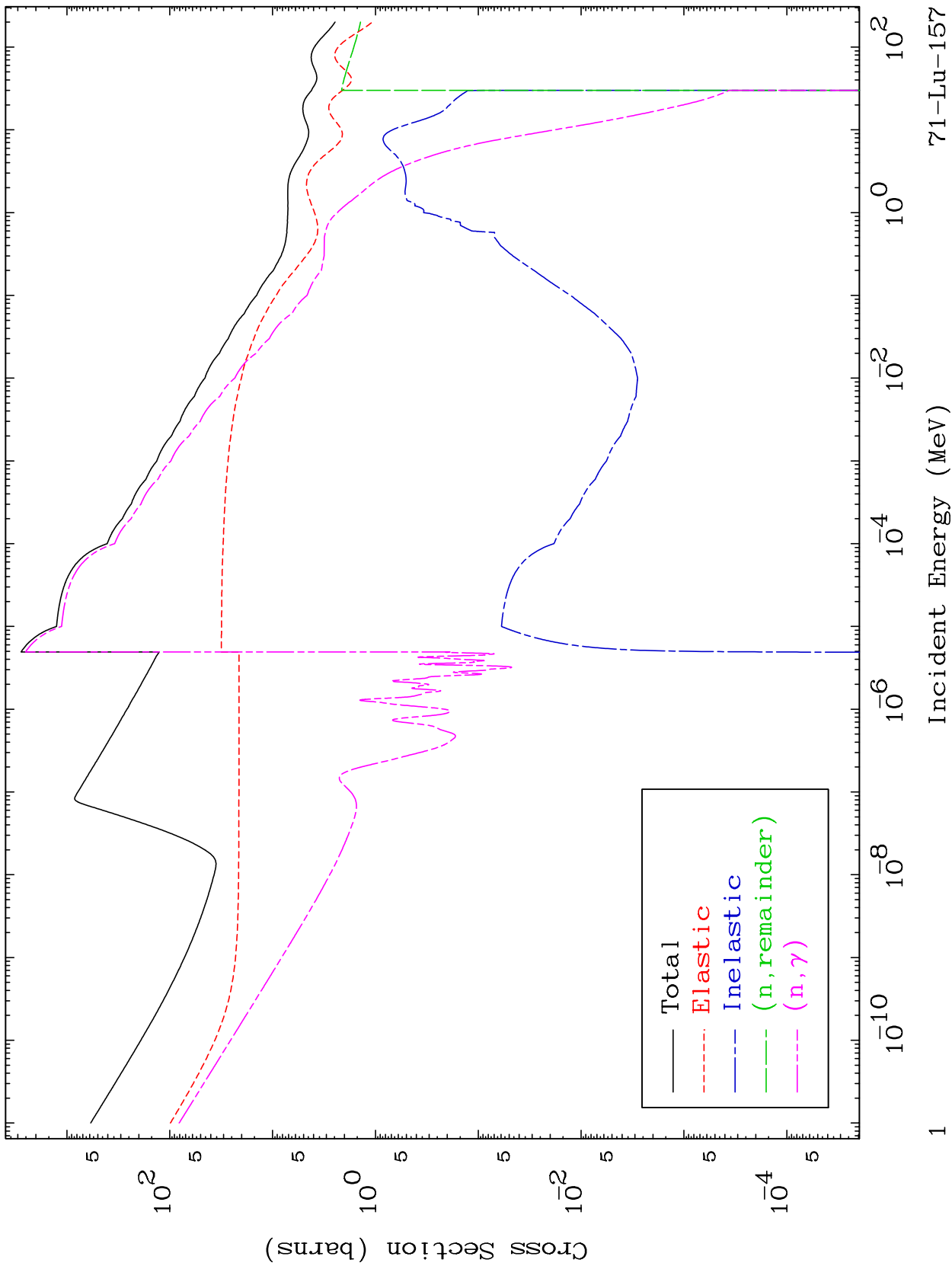
Press Mouse Button to Start

MAT 7072

Major

293 Kelvin Cross Sections

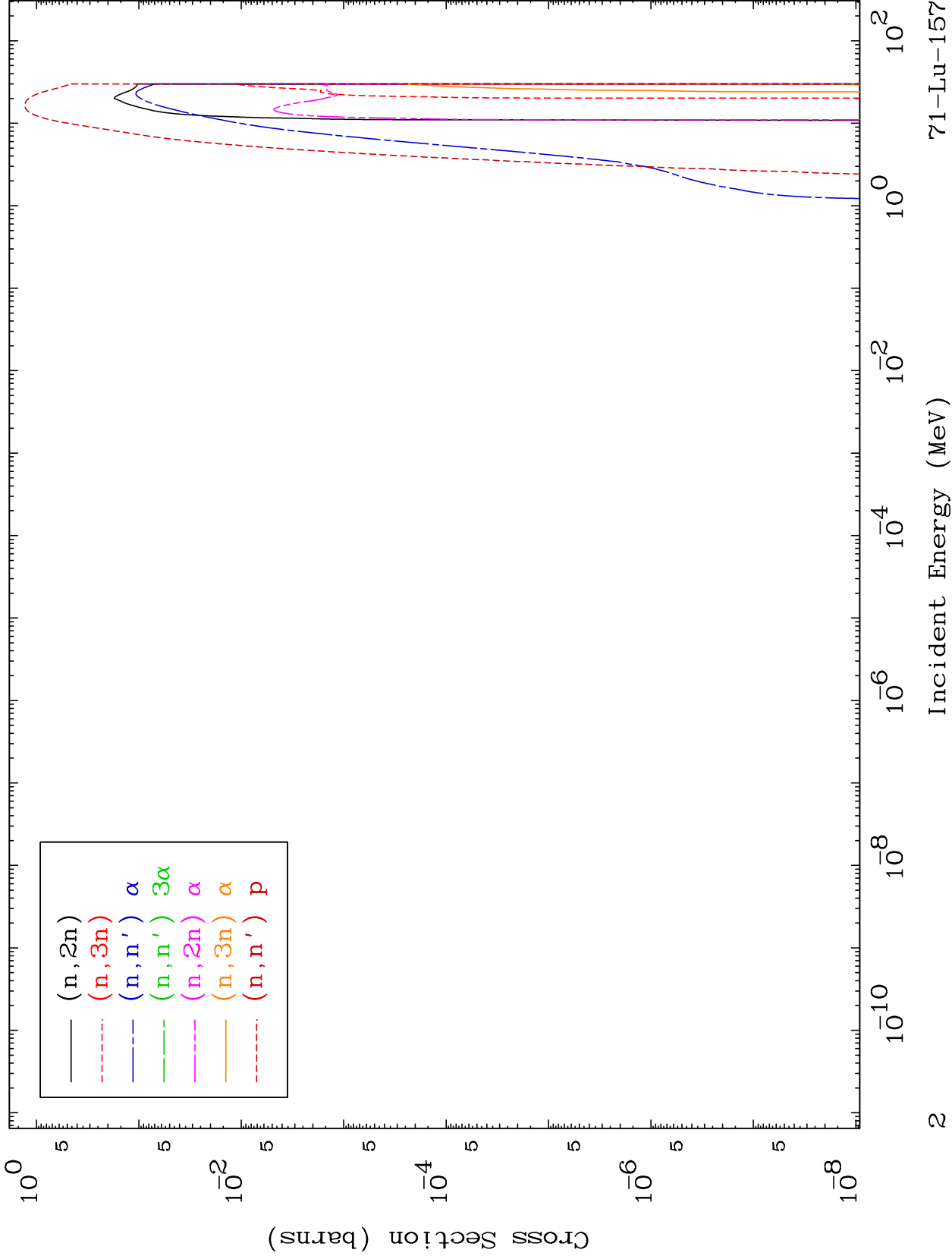
71-Lu-157



MAT 7072

Neutron Production
293 Kelvin Cross Sections

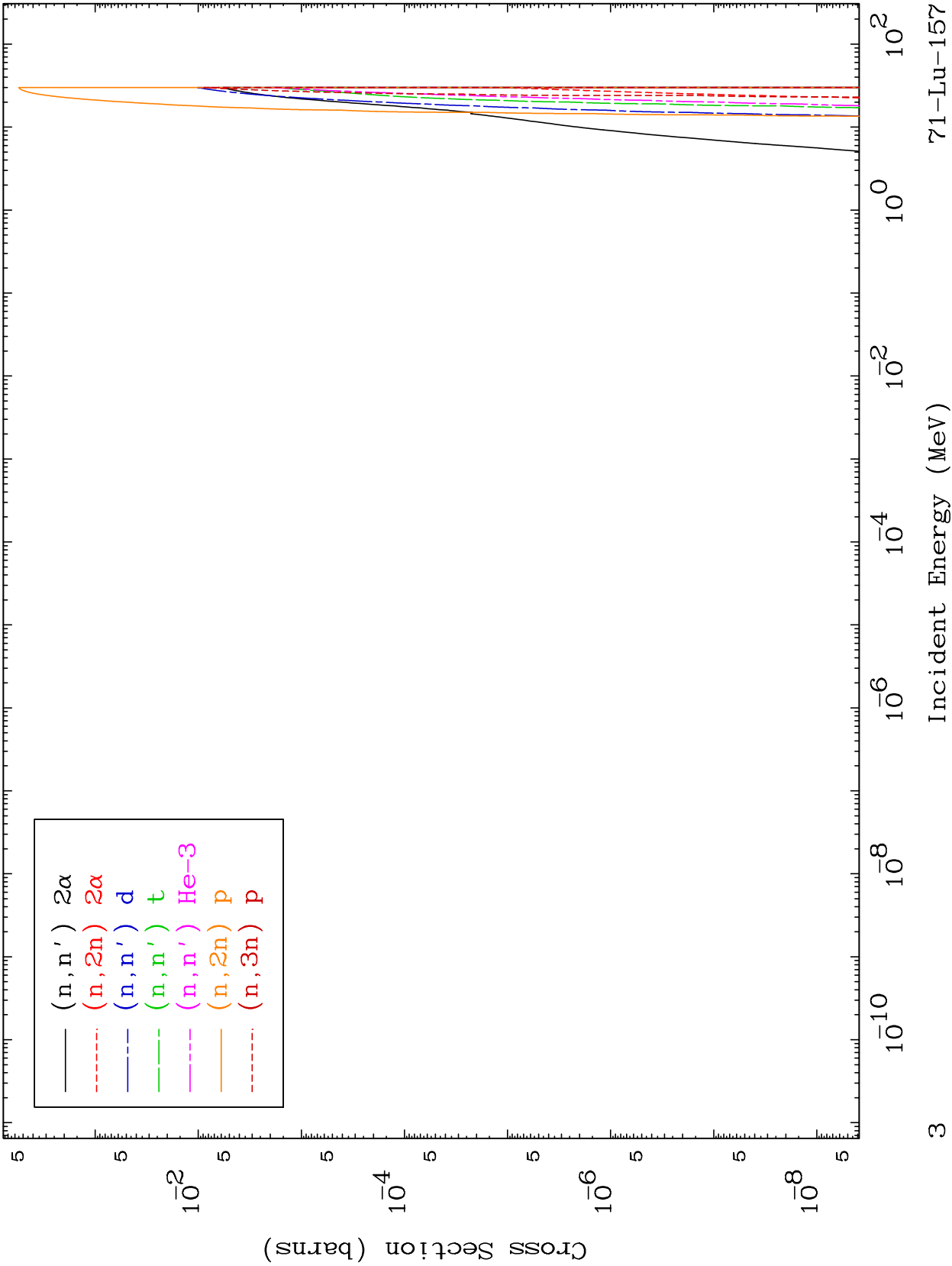
71-Lu-157



MAT 7072

Neutron Production
293 Kelvin Cross Sections

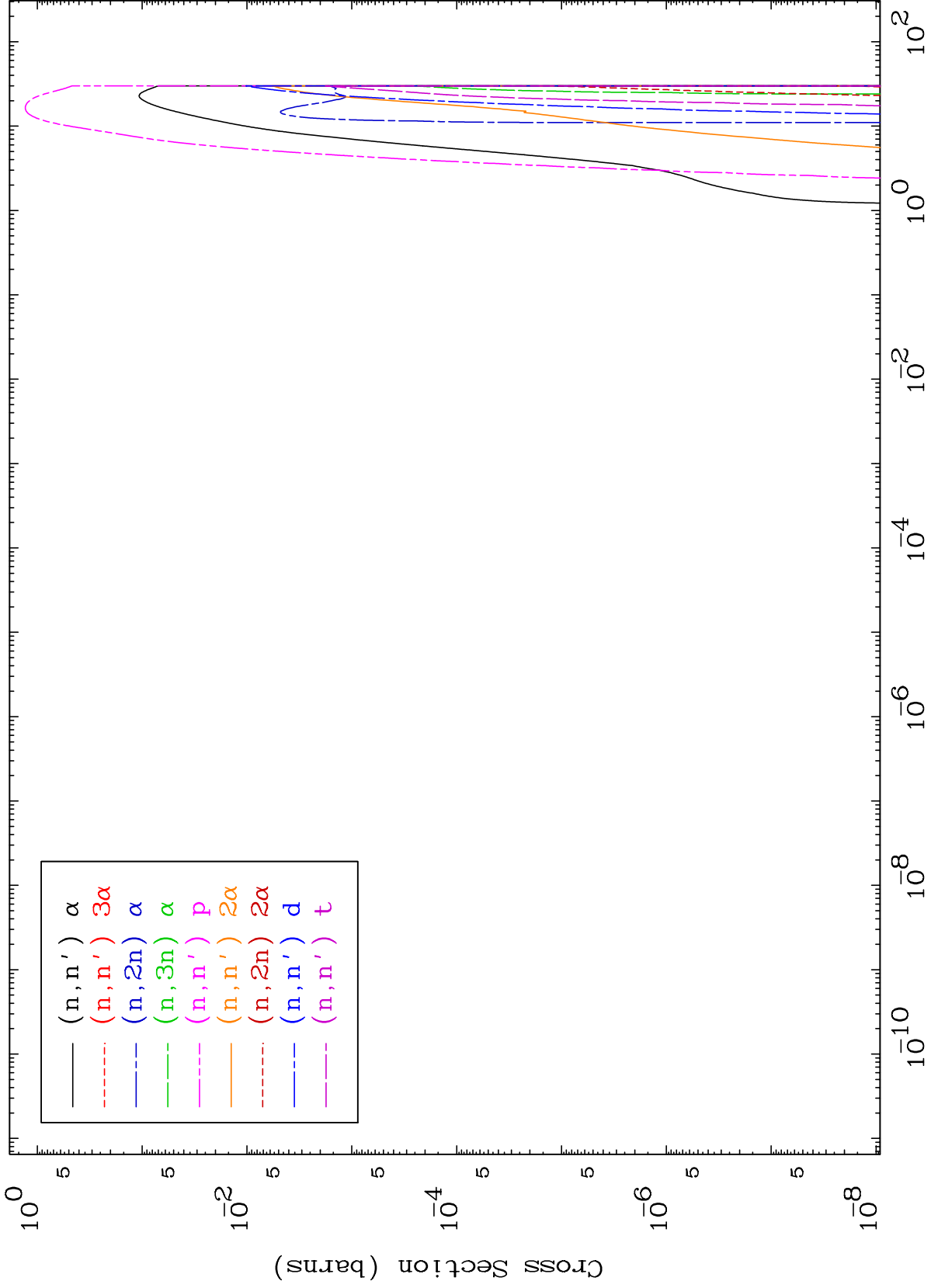
71-Lu-157

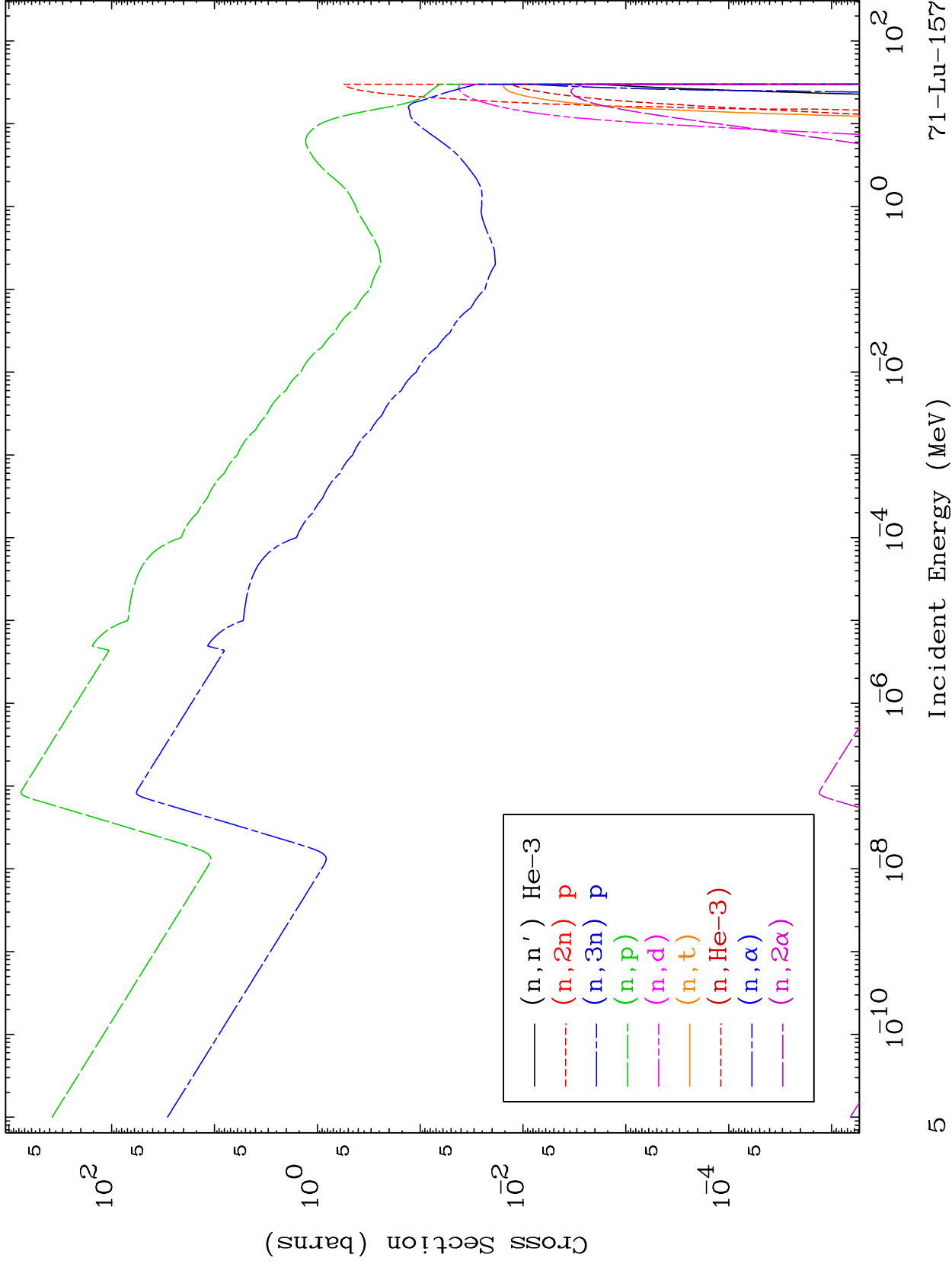


MAT 7072

Charged Particle
293 Kelvin Cross Sections

71-Lu-157

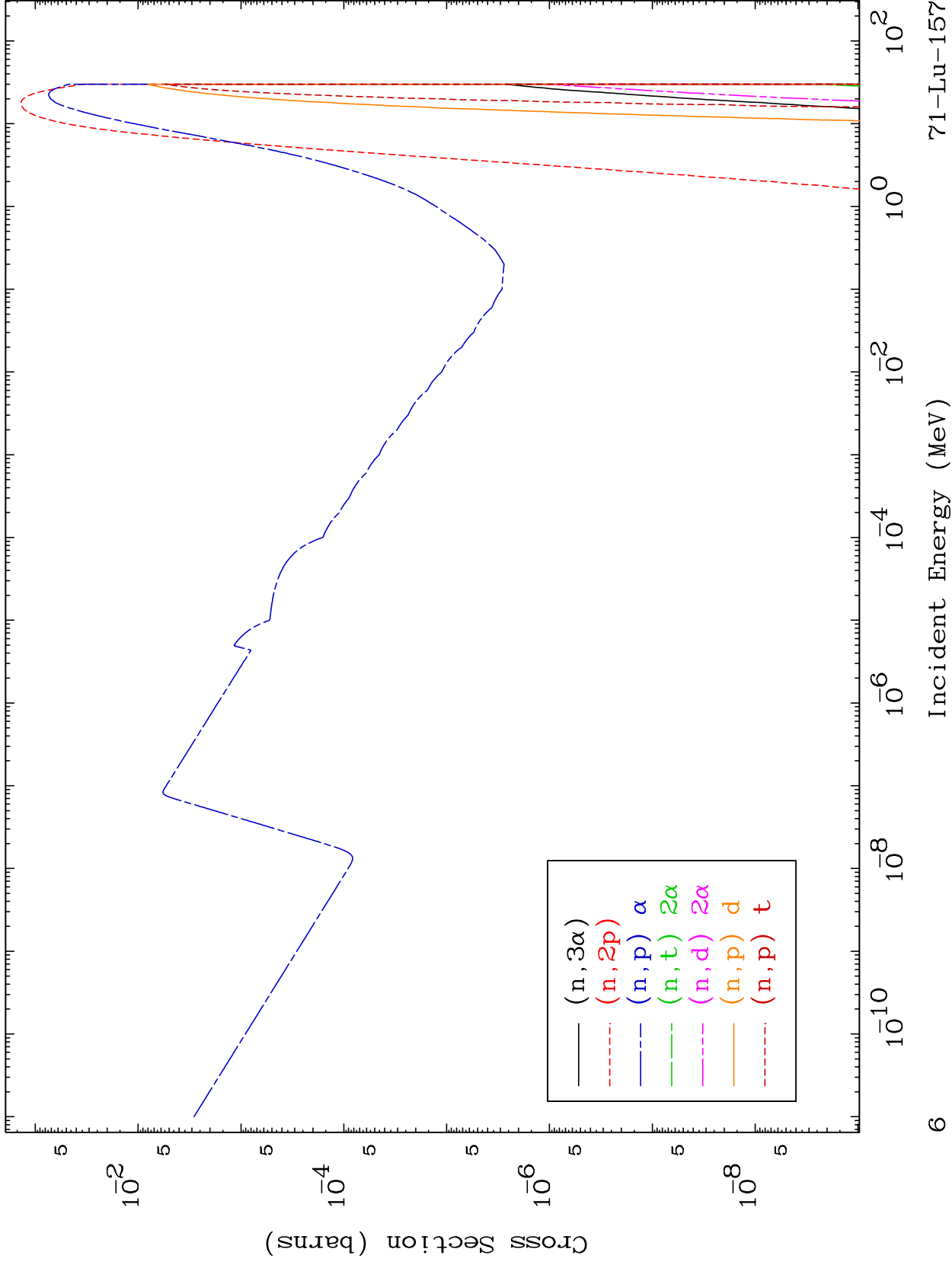


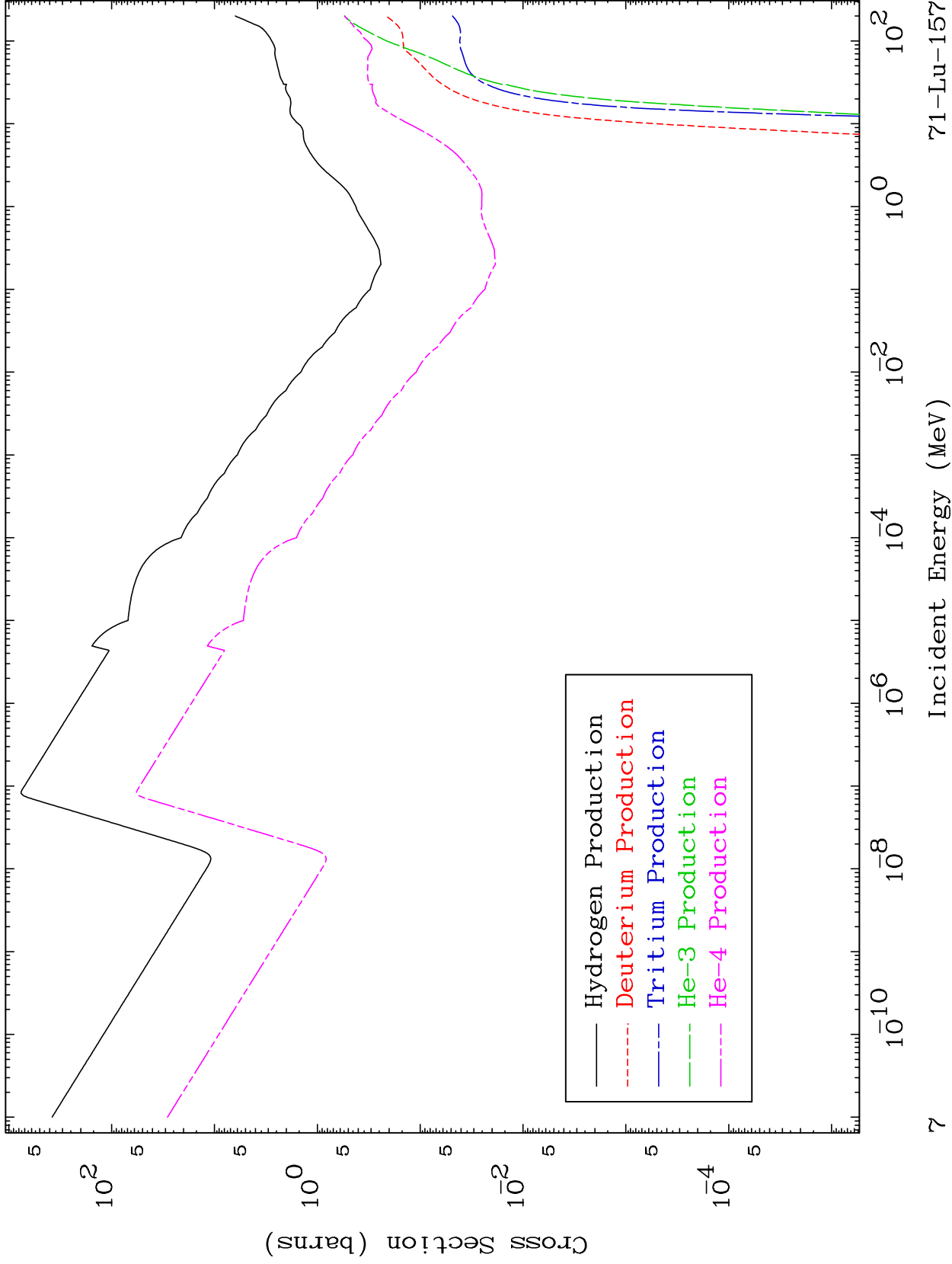


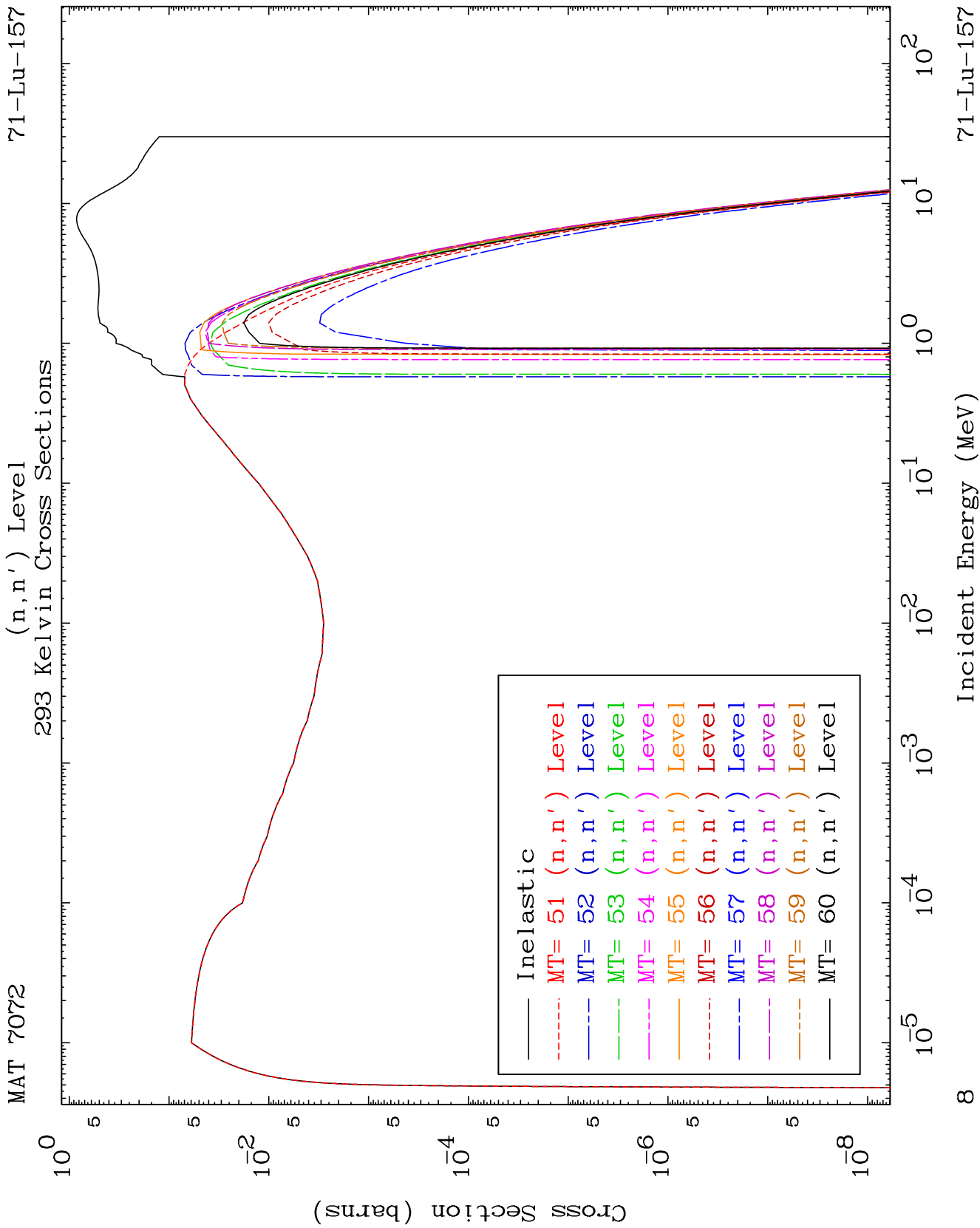
MAT 7072

Charged Particle
293 Kelvin Cross Sections

71-Lu-157





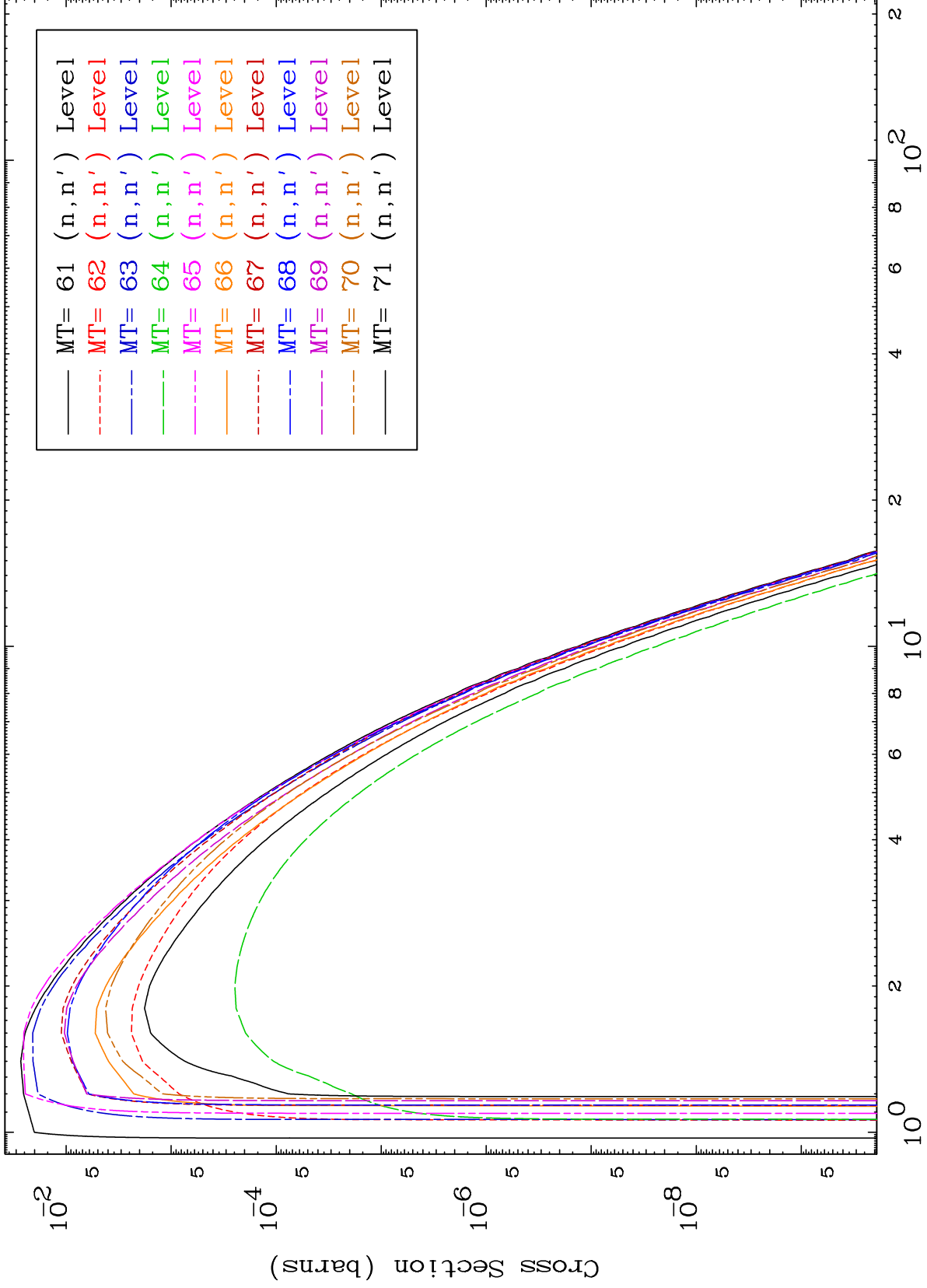


MAT 7072

(n,n') Level

71-Lu-157

293 Kelvin Cross Sections



Incident Energy (MeV)

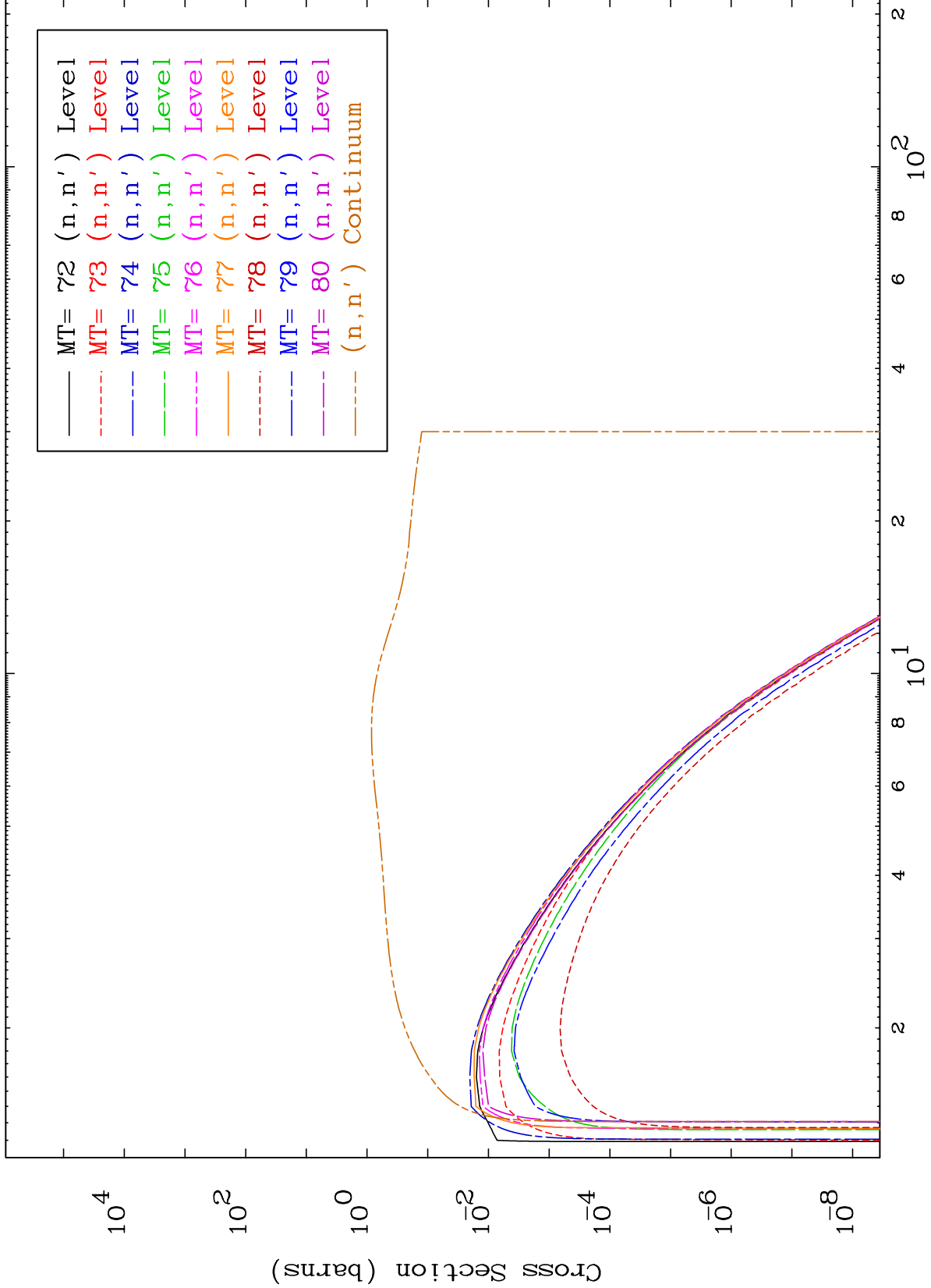
71-Lu-157

MAT 7072

(n,n') Level

293 Kelvin Cross Sections

71-Lu-157



10

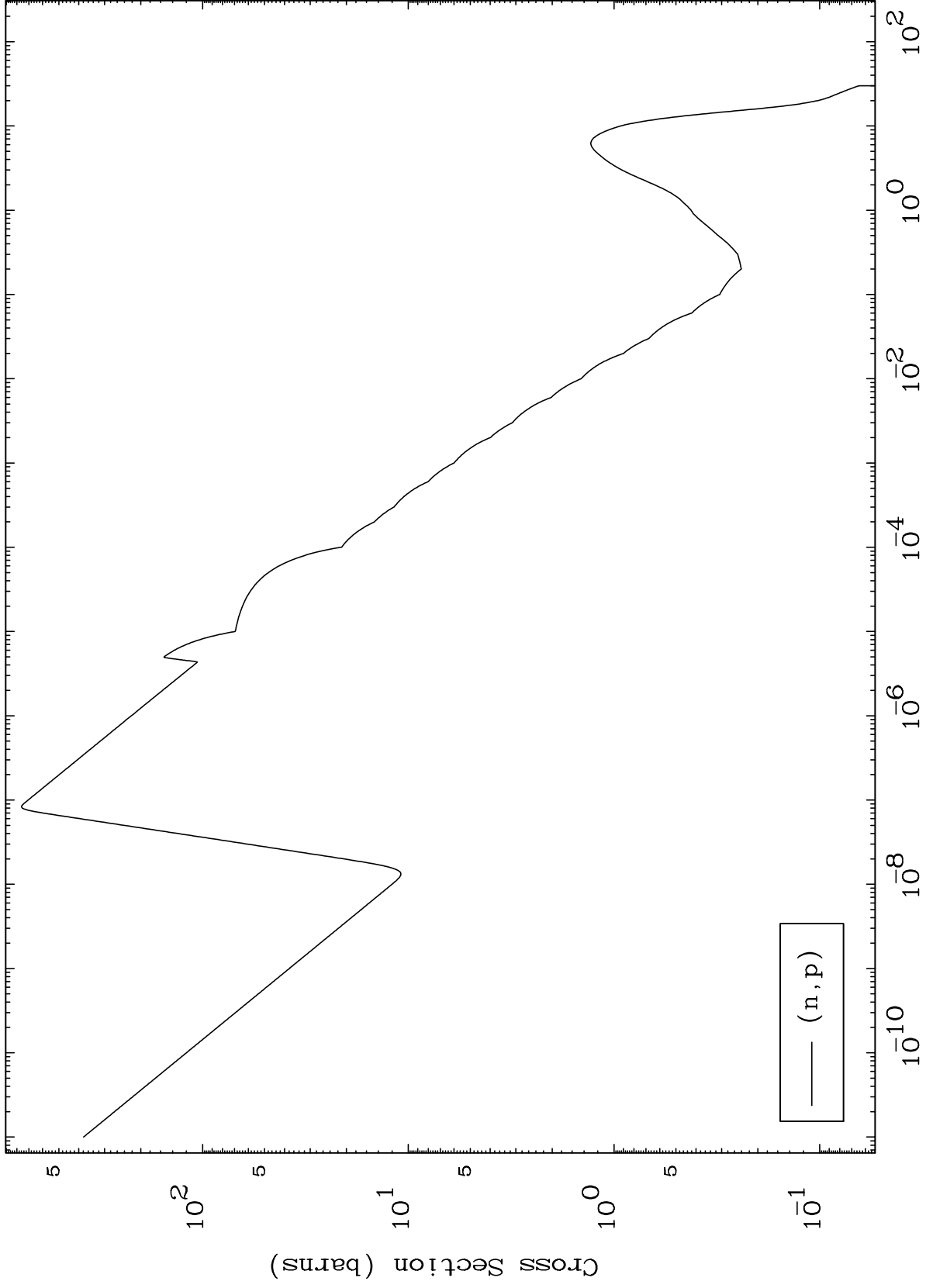
Incident Energy (MeV)

71-Lu-157

MAT 7072

(n,p) Levels
293 Kelvin Cross Sections

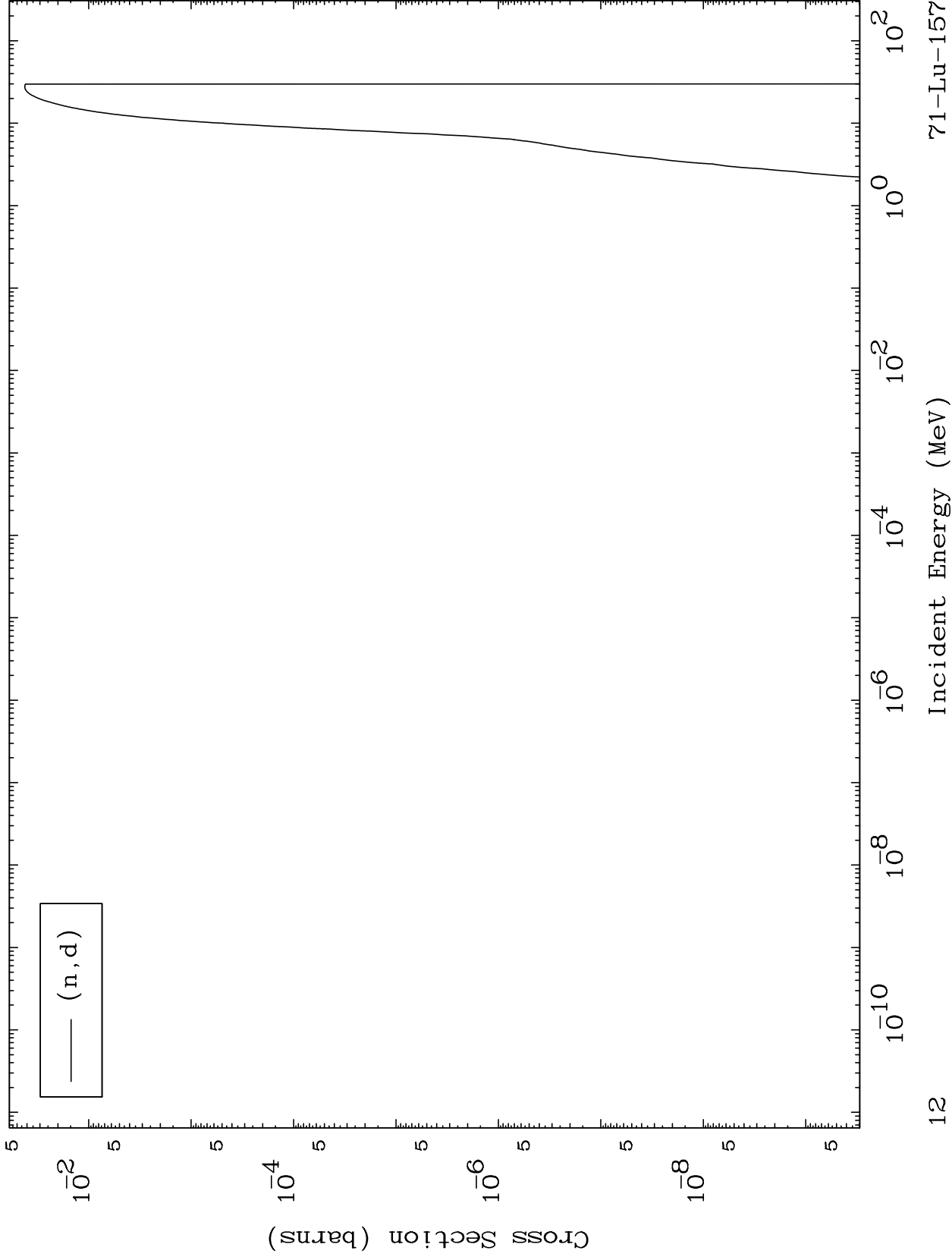
71-Lu-157



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(n,d) Levels
293 Kelvin Cross Sections

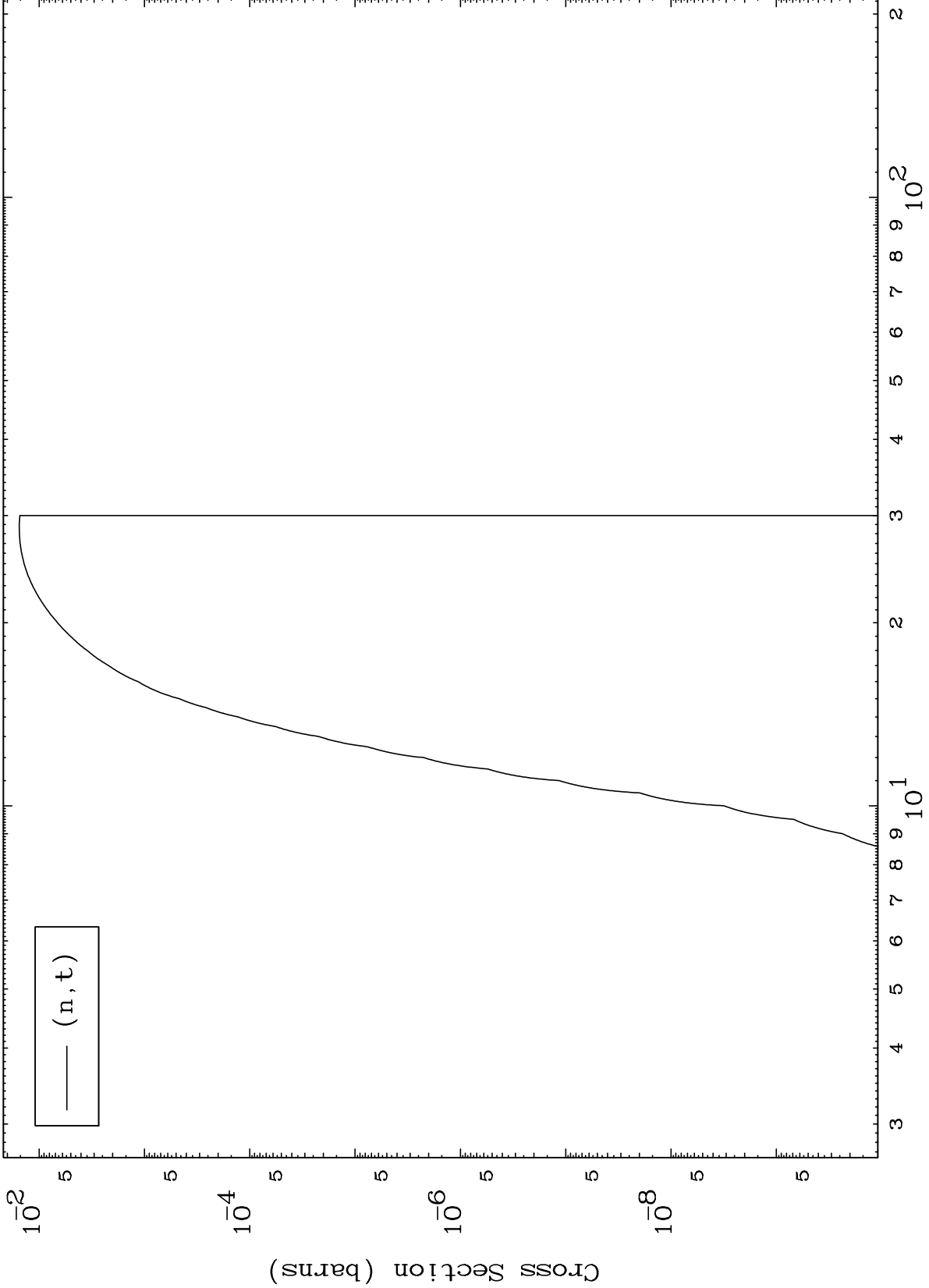
71-Lu-157



MAT 7072

(n,t) Levels
293 Kelvin Cross Sections

71-Lu-157



13

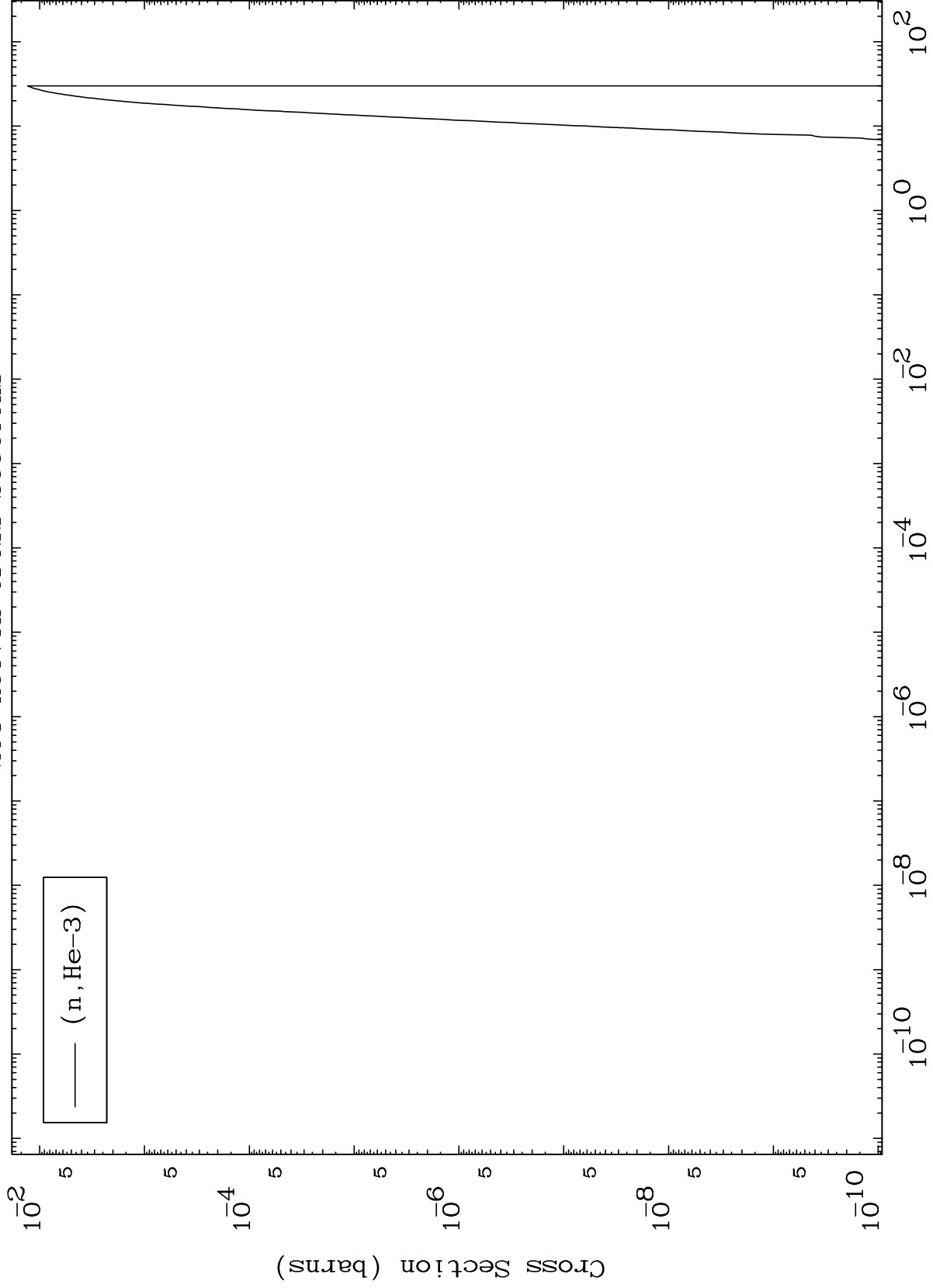
Incident Energy (MeV)

71-Lu-157

MAT 7072

(n,He3) Levels
293 Kelvin Cross Sections

71-Lu-157



14

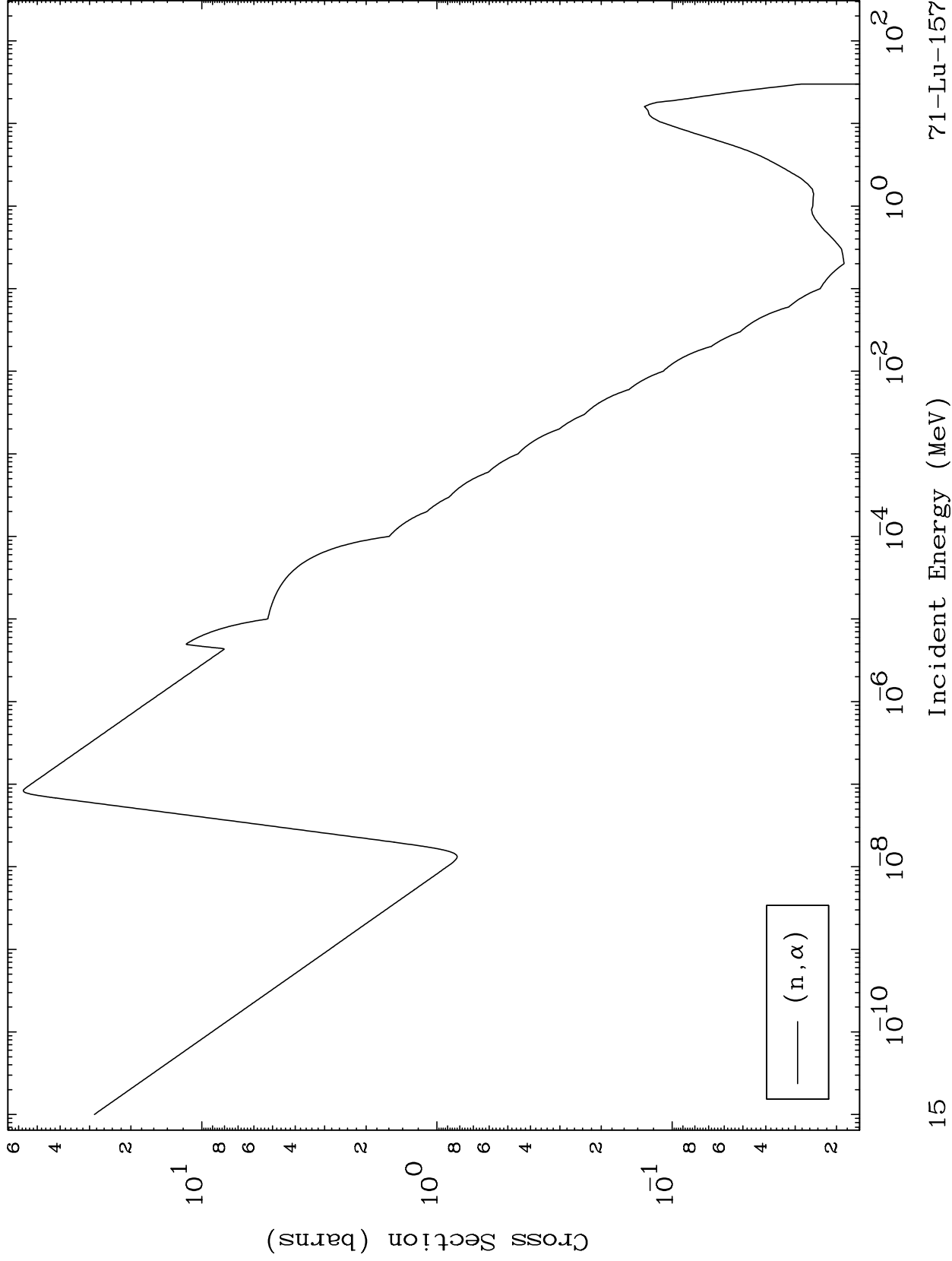
Incident Energy (MeV)

71-Lu-157

MAT 7072

(n, α) Levels
293 Kelvin Cross Sections

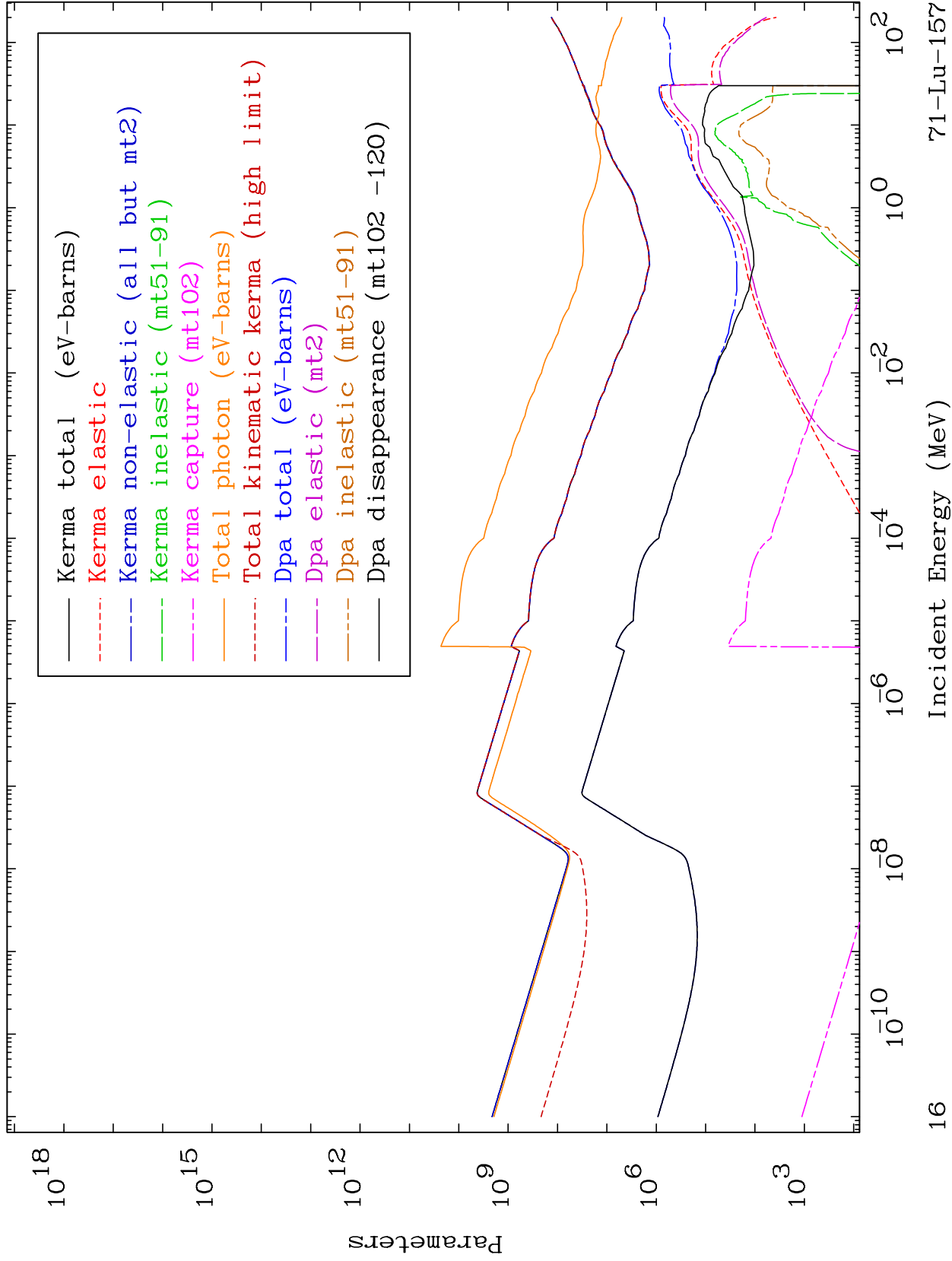
71-Lu-157

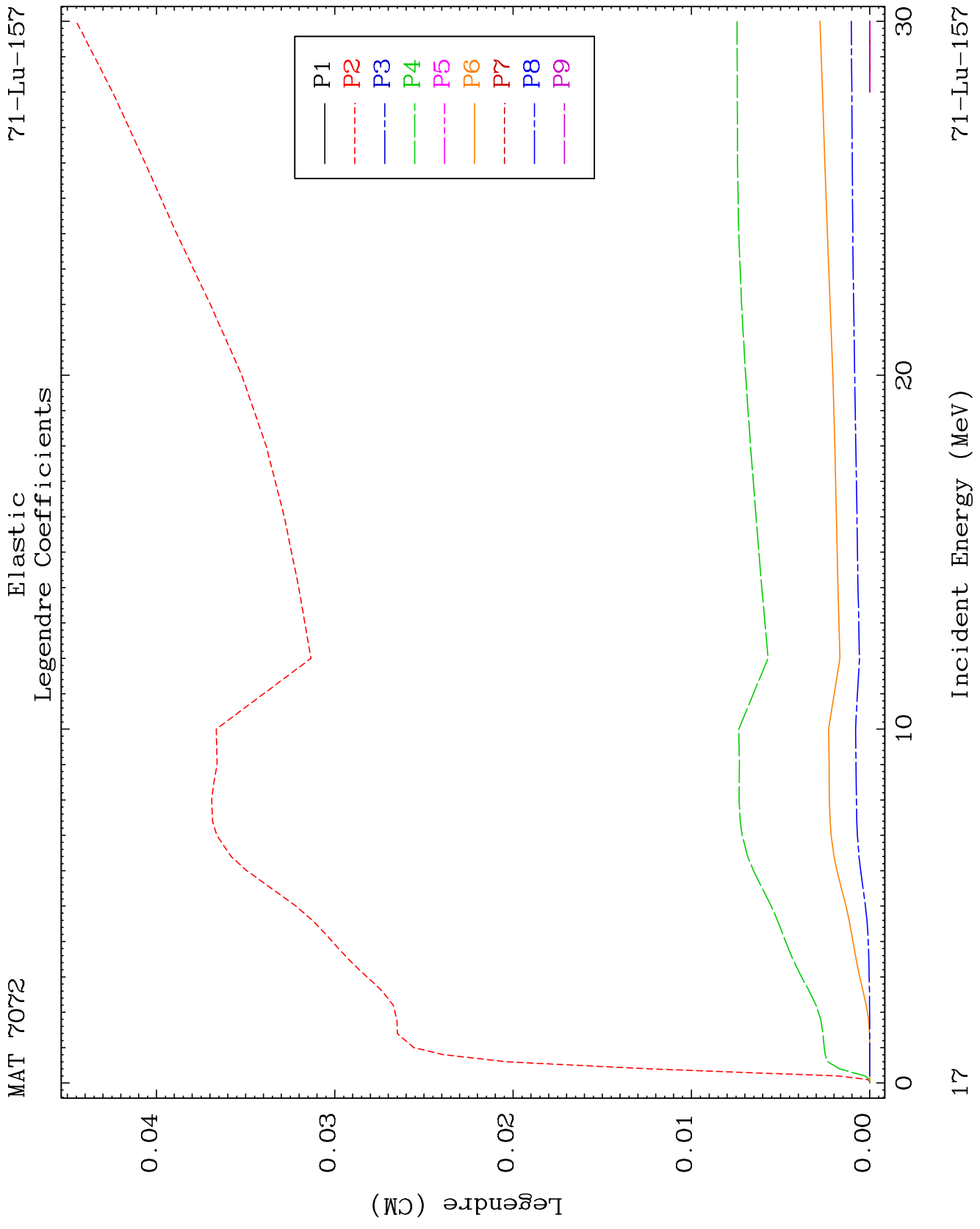


15

Incident Energy (MeV)

71-Lu-157





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$^{71}\text{Lu-157}$

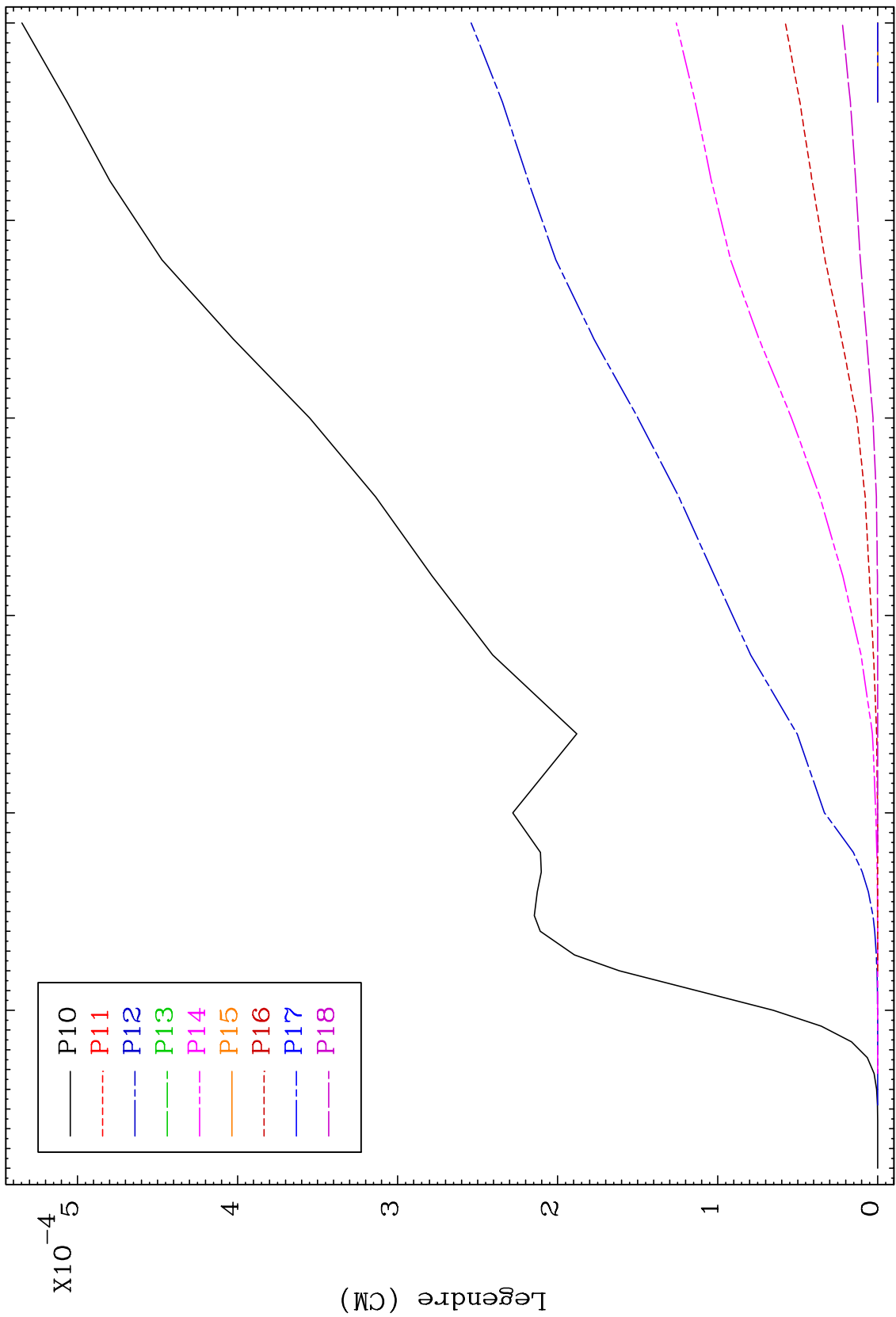
17

$^{71}\text{Lu-157}$

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Elastic Legendre Coefficients

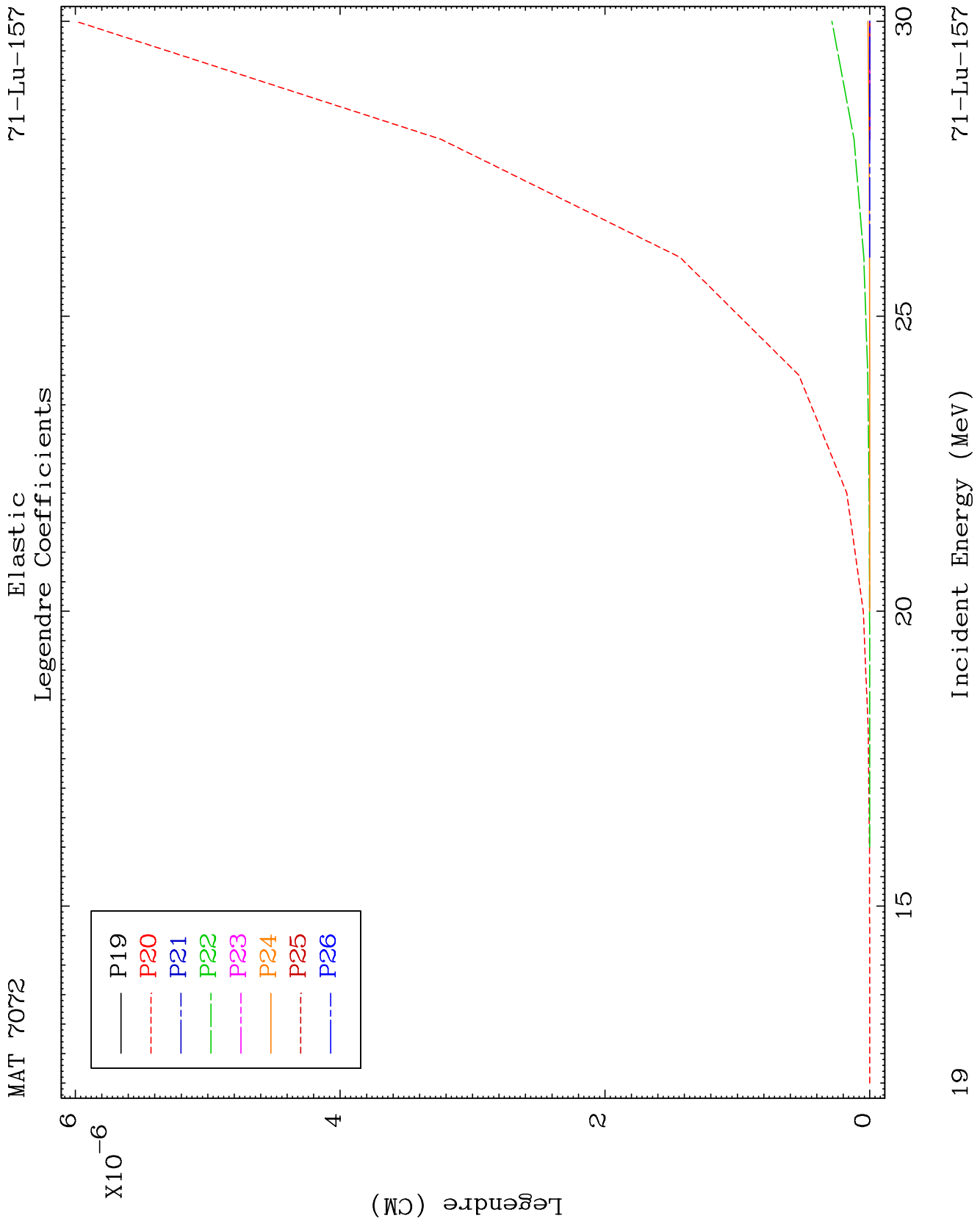
71-Lu-157



18

Incident Energy (MeV)

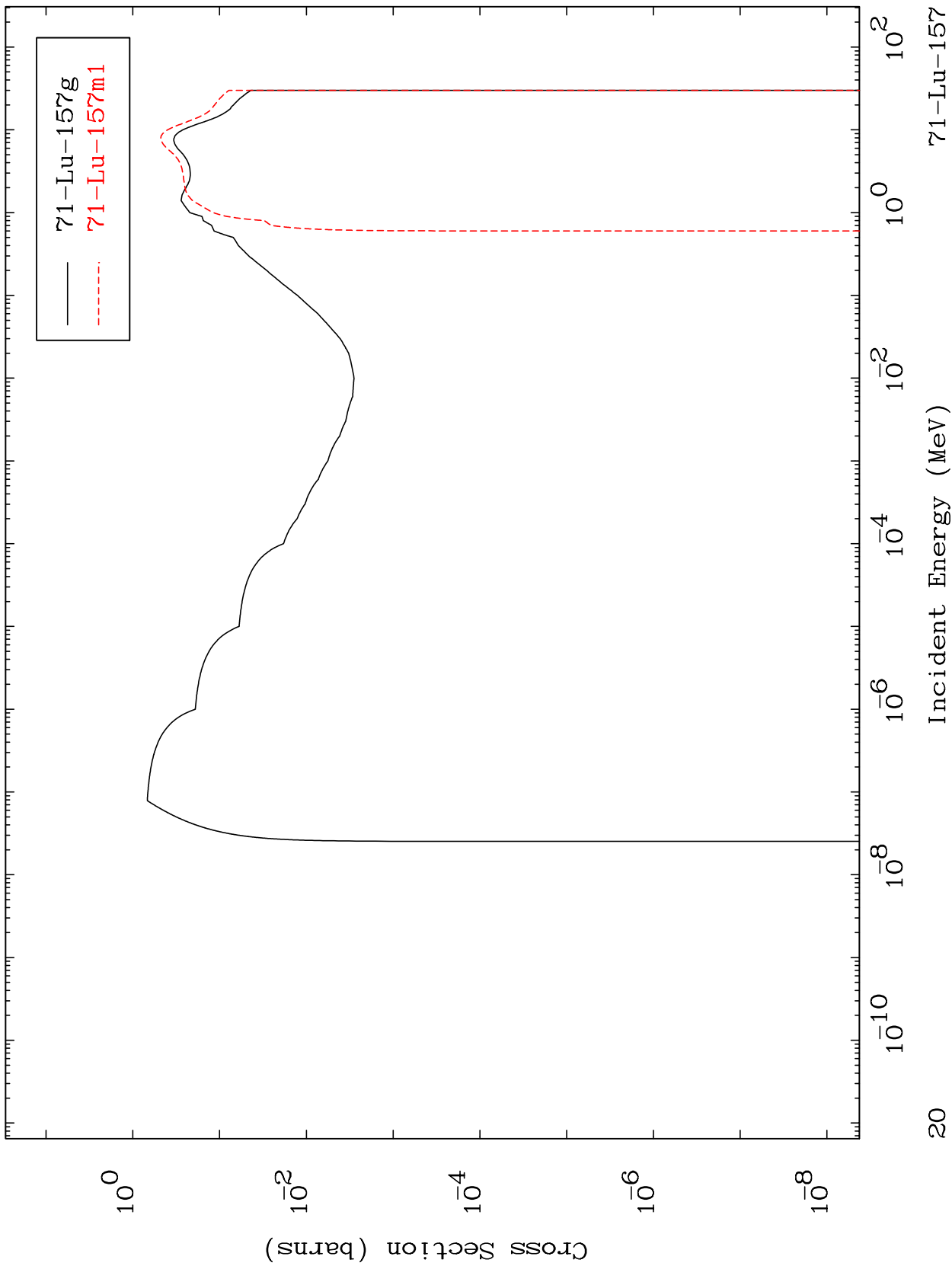
71-Lu-157



MAT 7072

71-Lu-157

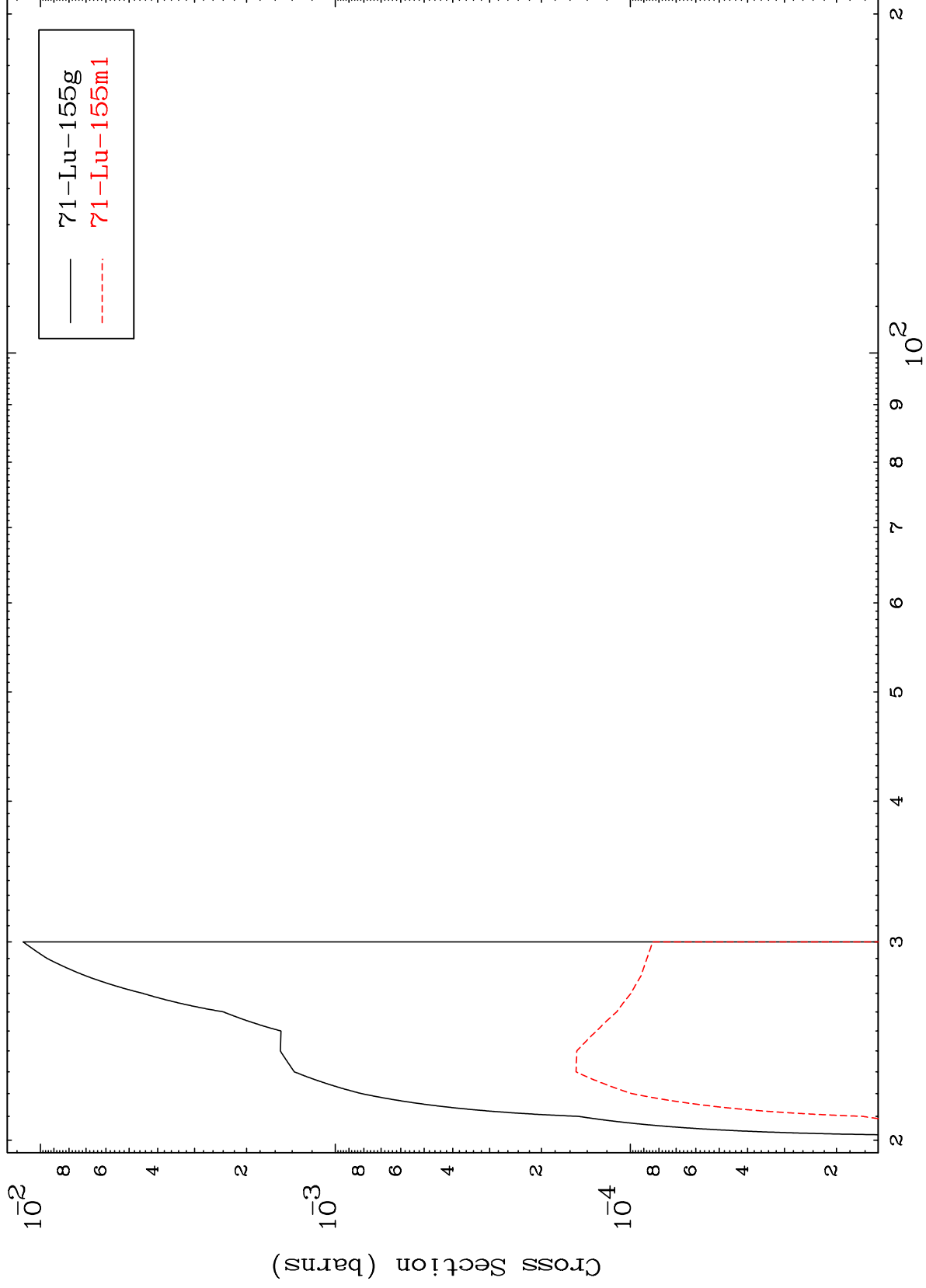
Inelastic
Radionuclide Production Cross Section



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71-Lu-157

(n,3n)
Radionuclide Production Cross Section



71-Lu-157

Incident Energy (MeV)

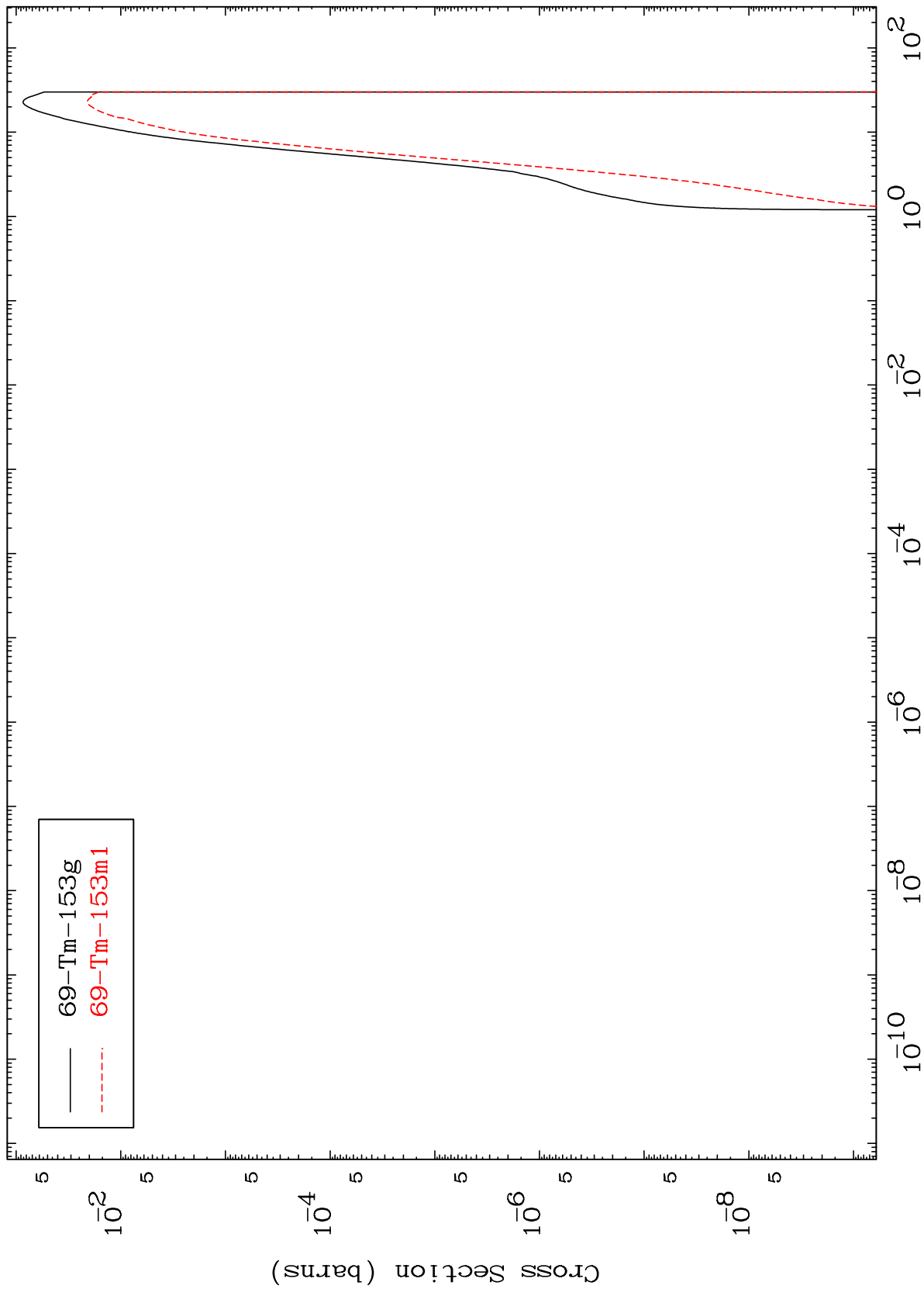
21

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$(n, n') \alpha$

71-Lu-157

Radionuclide Production Cross Section

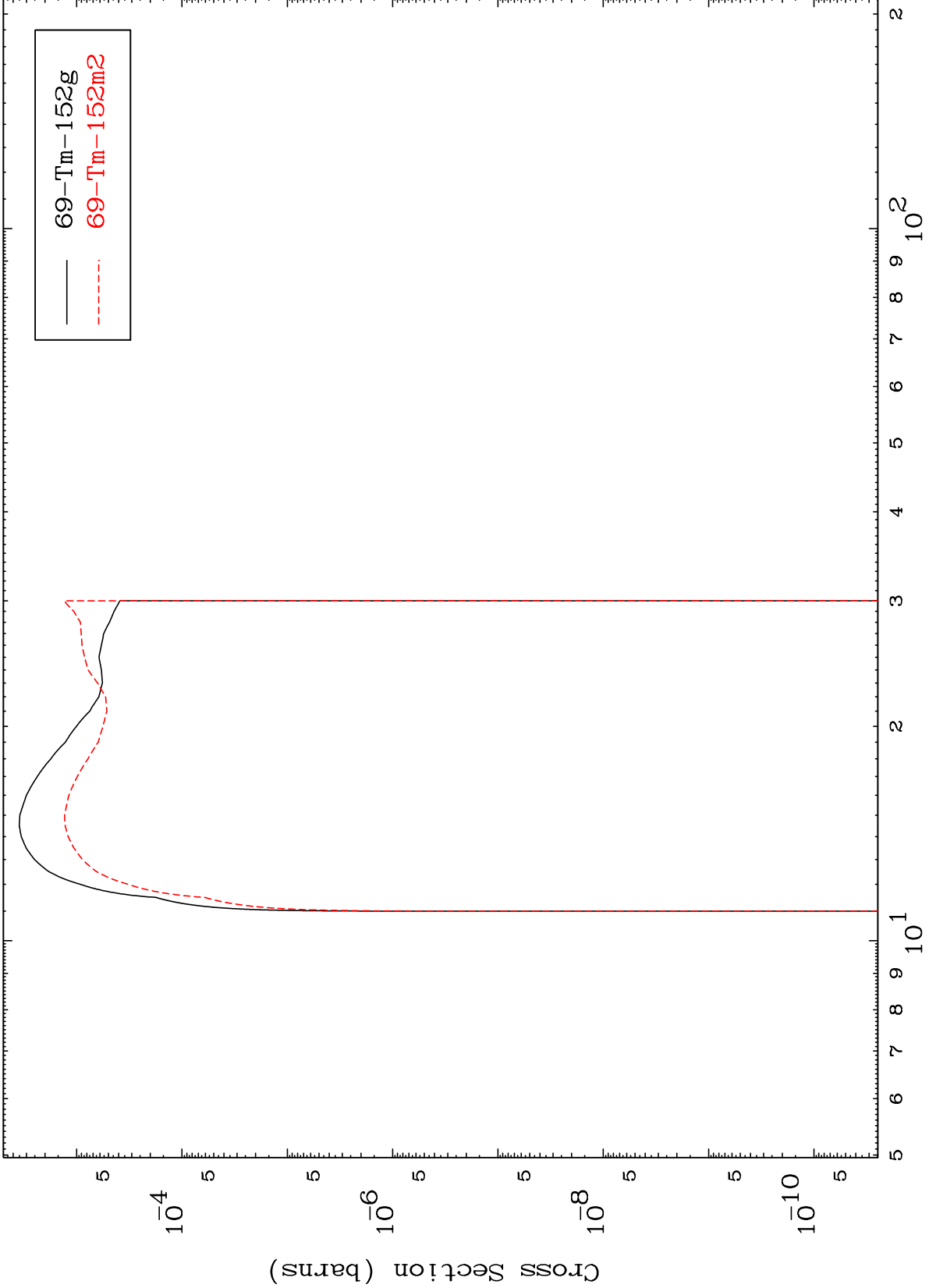


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$(n,2n) \alpha$

71-Lu-157

Radionuclide Production Cross Section



23

Incident Energy (MeV)

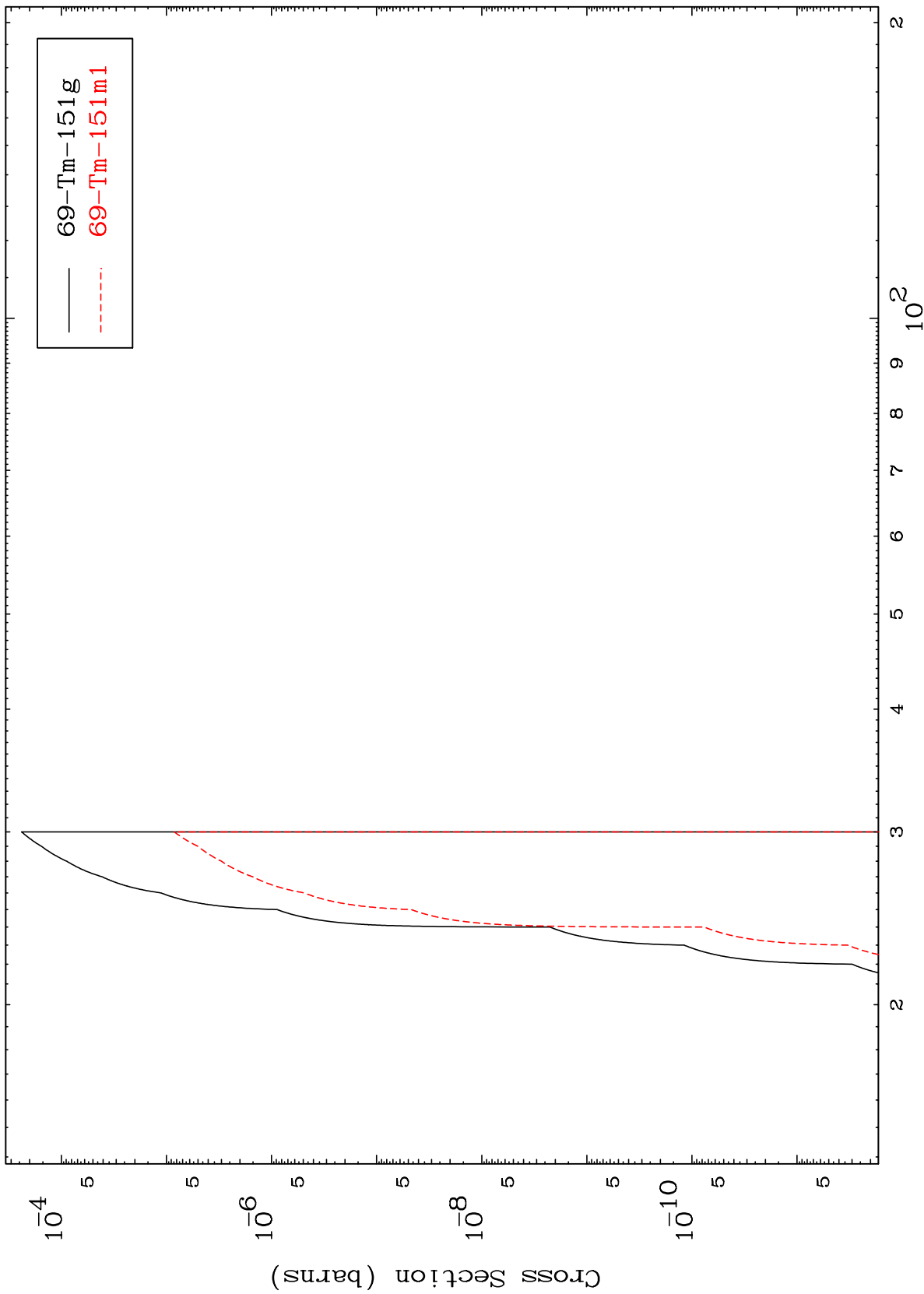
71-Lu-157

MAT 7072

(n,3n) α

71-Lu-157

Radionuclide Production Cross Section



69-Tm-151g
69-Tm-151m1

24

Incident Energy (MeV)

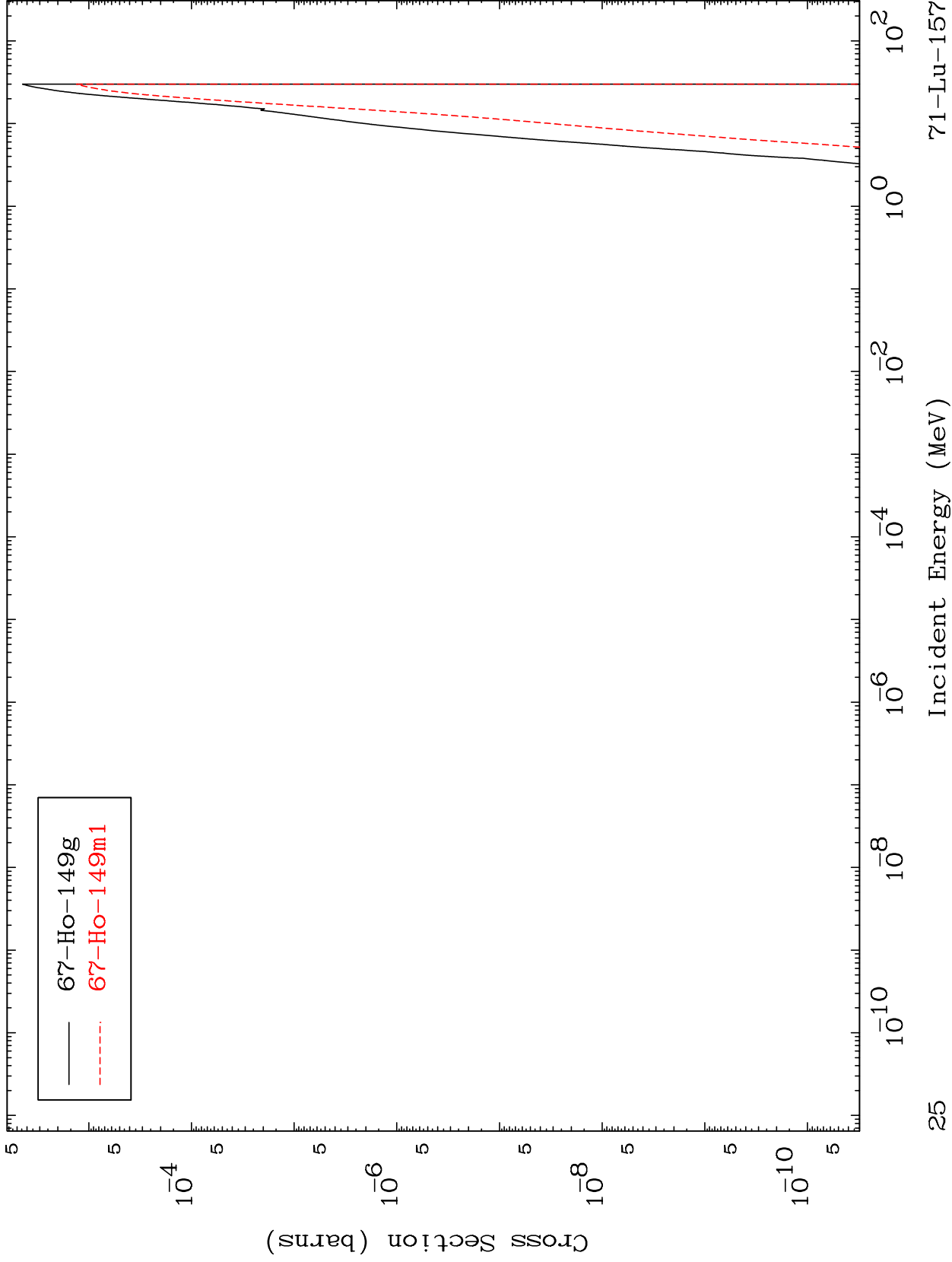
71-Lu-157

MAT 7072

(n,n') 2α

71-Lu-157

Radionuclide Production Cross Section



25

Incident Energy (MeV)

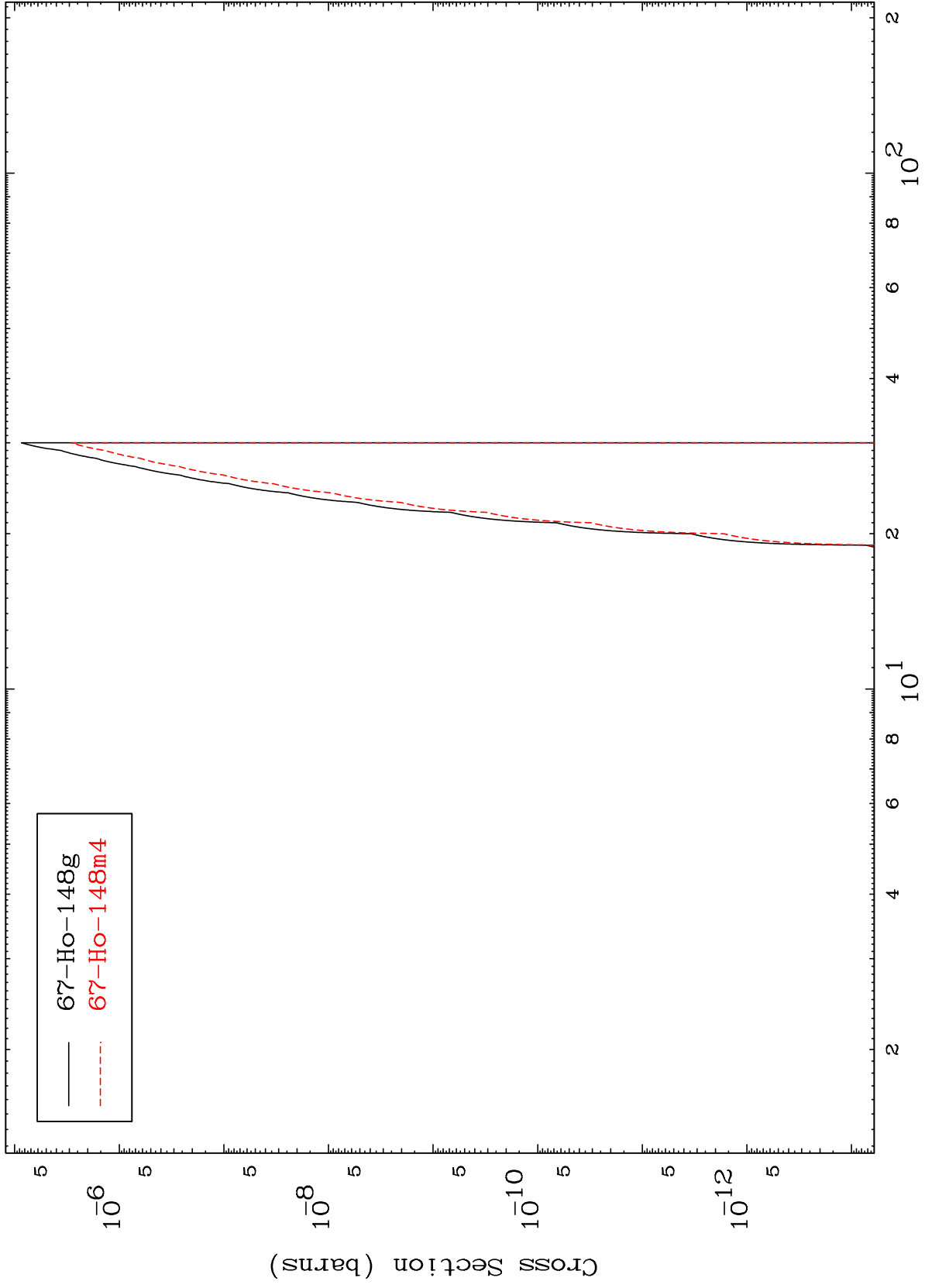
71-Lu-157

MAT 7072

$(n,2n) 2\alpha$

$^{71}\text{Lu-157}$

Radionuclide Production Cross Section

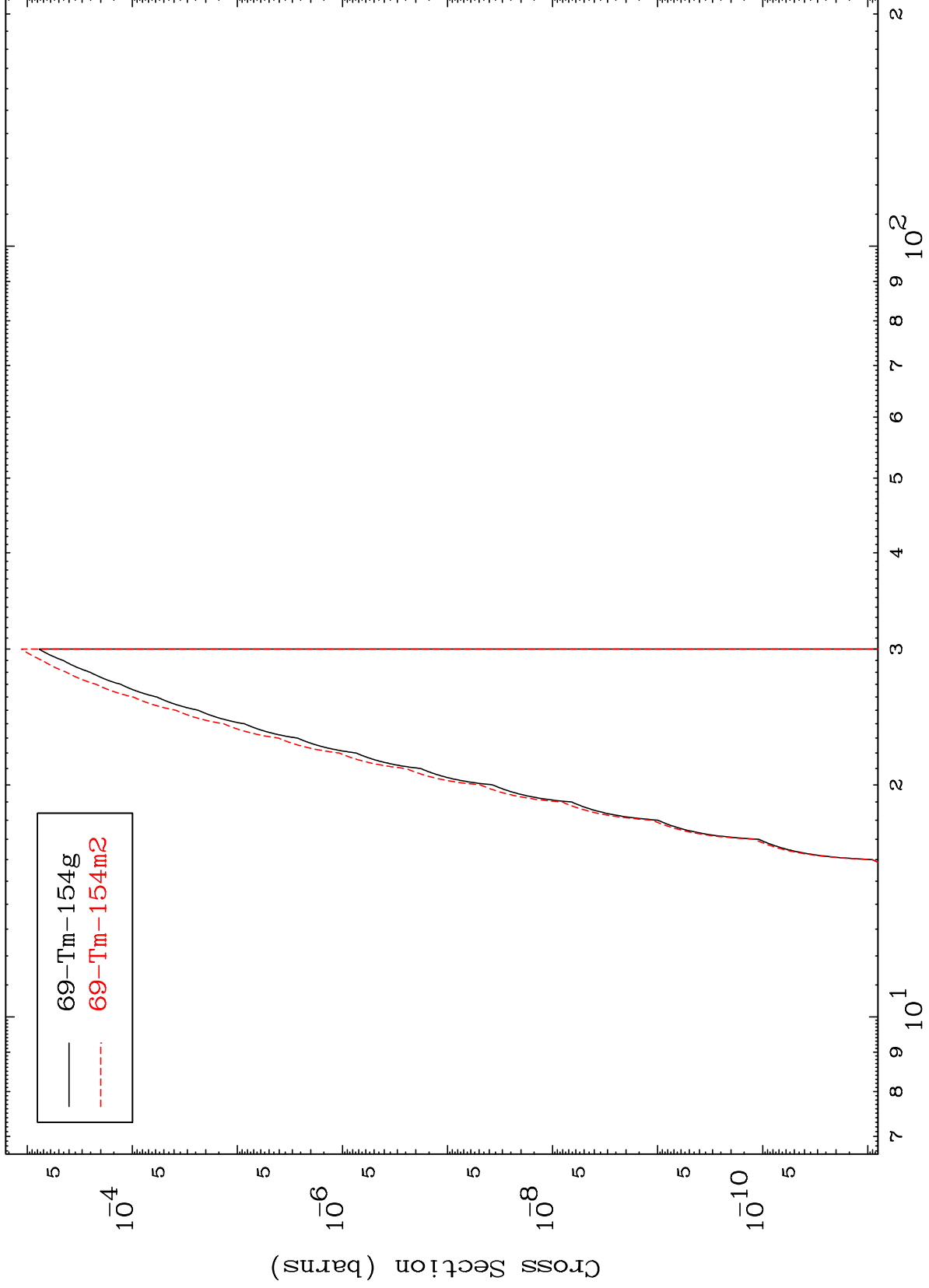


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(n,n') He-3

71-Lu-157

Radionuclide Production Cross Section



27

Incident Energy (MeV)

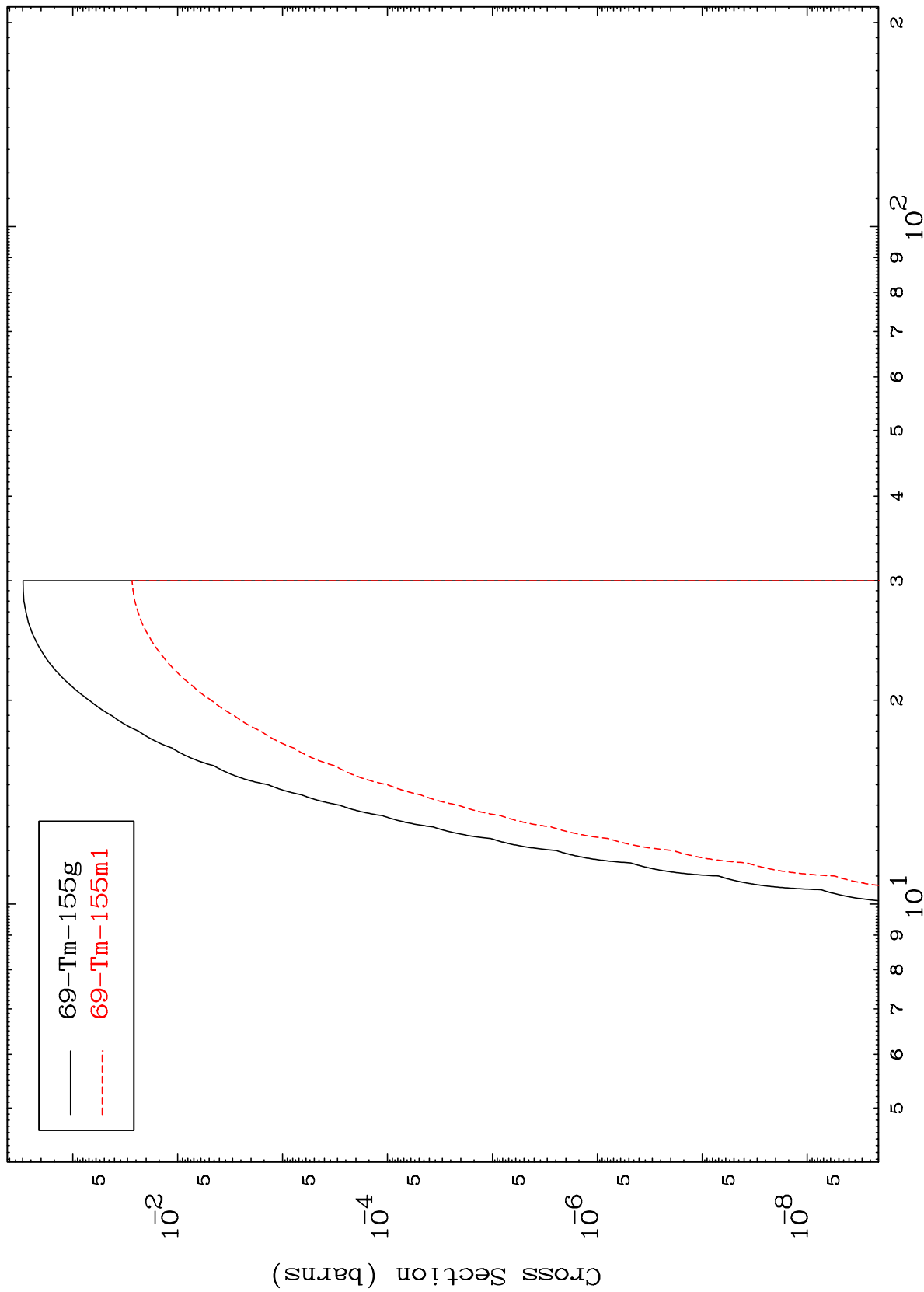
71-Lu-157

MAT 7072

(n,2n) p

71-Lu-157

Radionuclide Production Cross Section



69-Tm-155g
69-Tm-155m1

28

Incident Energy (MeV)

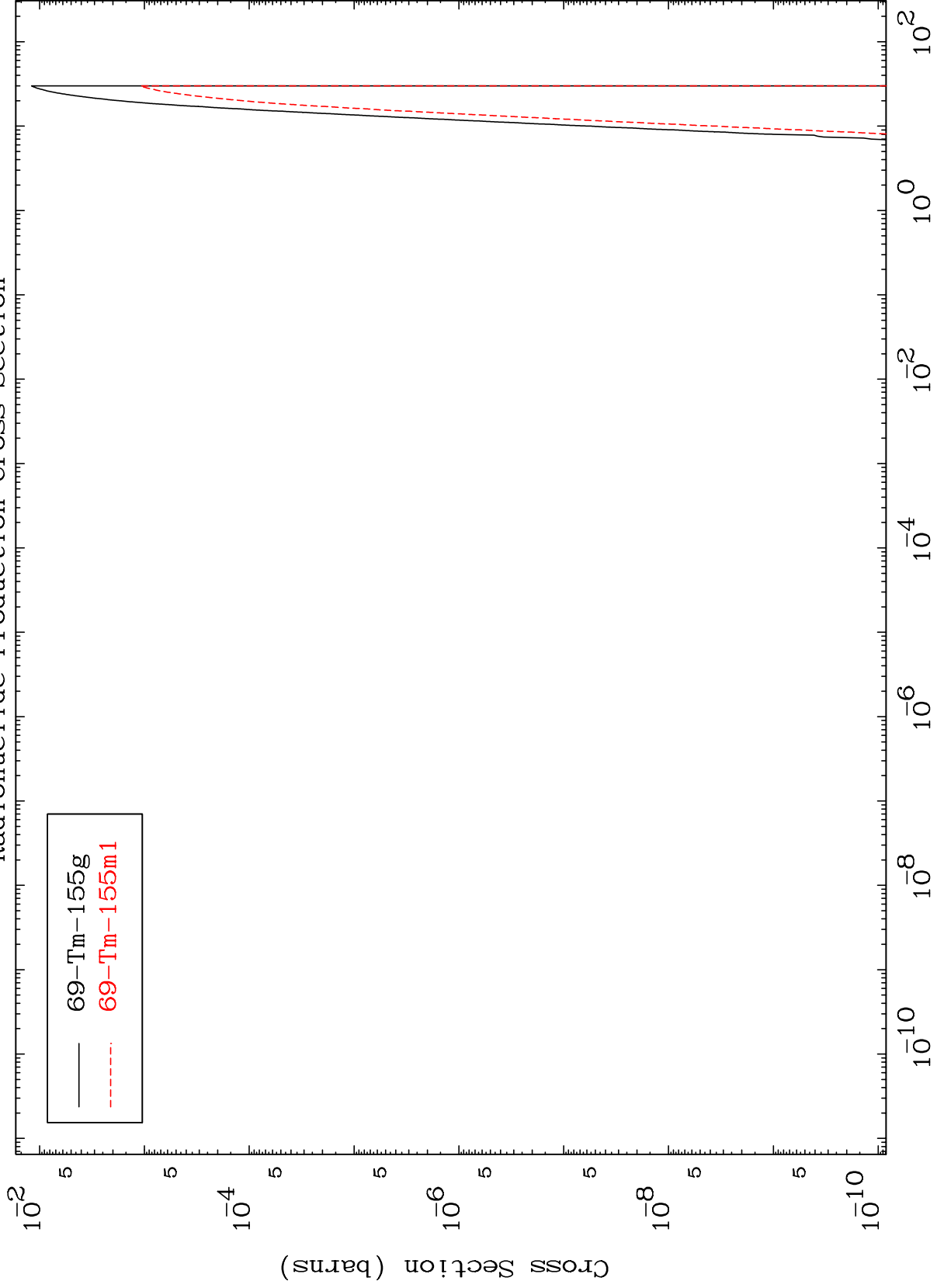
71-Lu-157

MAT 7072

(n,He-3)

71-Lu-157

Radionuclide Production Cross Section



29

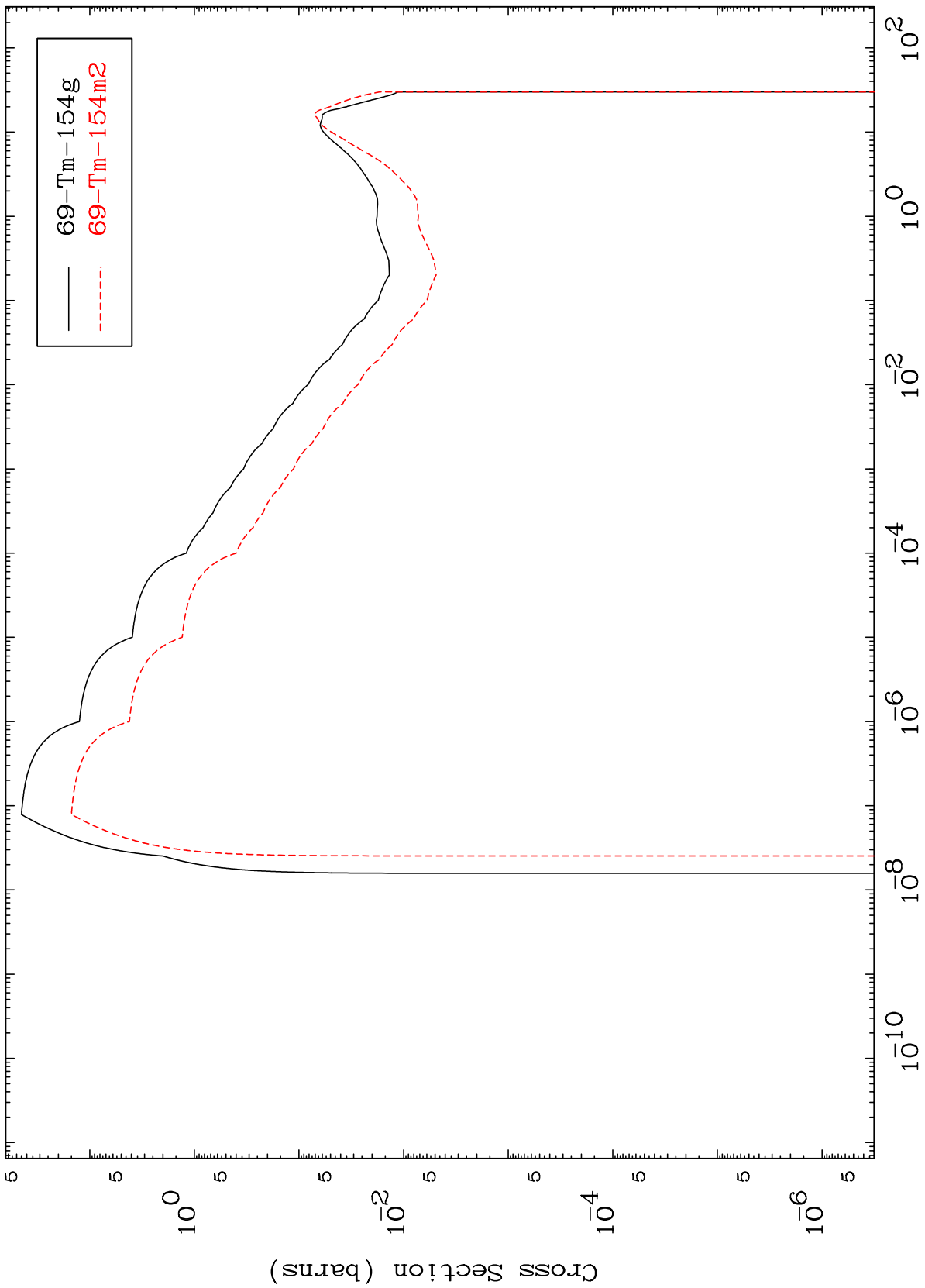
Incident Energy (MeV)

71-Lu-157

MAT 7072

71-Lu-157

(n, α)
Radionuclide Production Cross Section



30

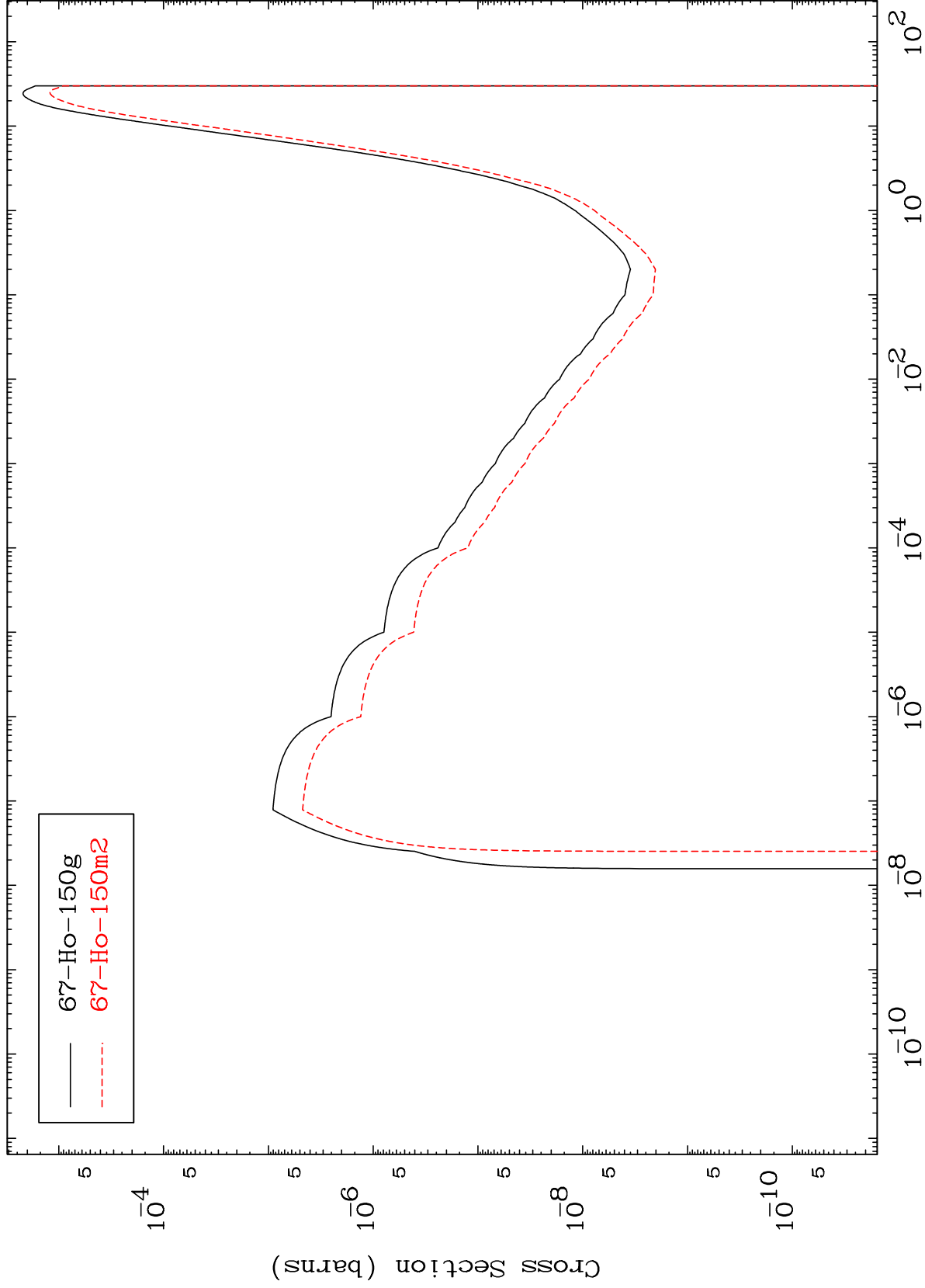
71-Lu-157

Incident Energy (MeV)

MAT 7072

71-Lu-157

Radionuclide Production Cross Section
(n,2α)

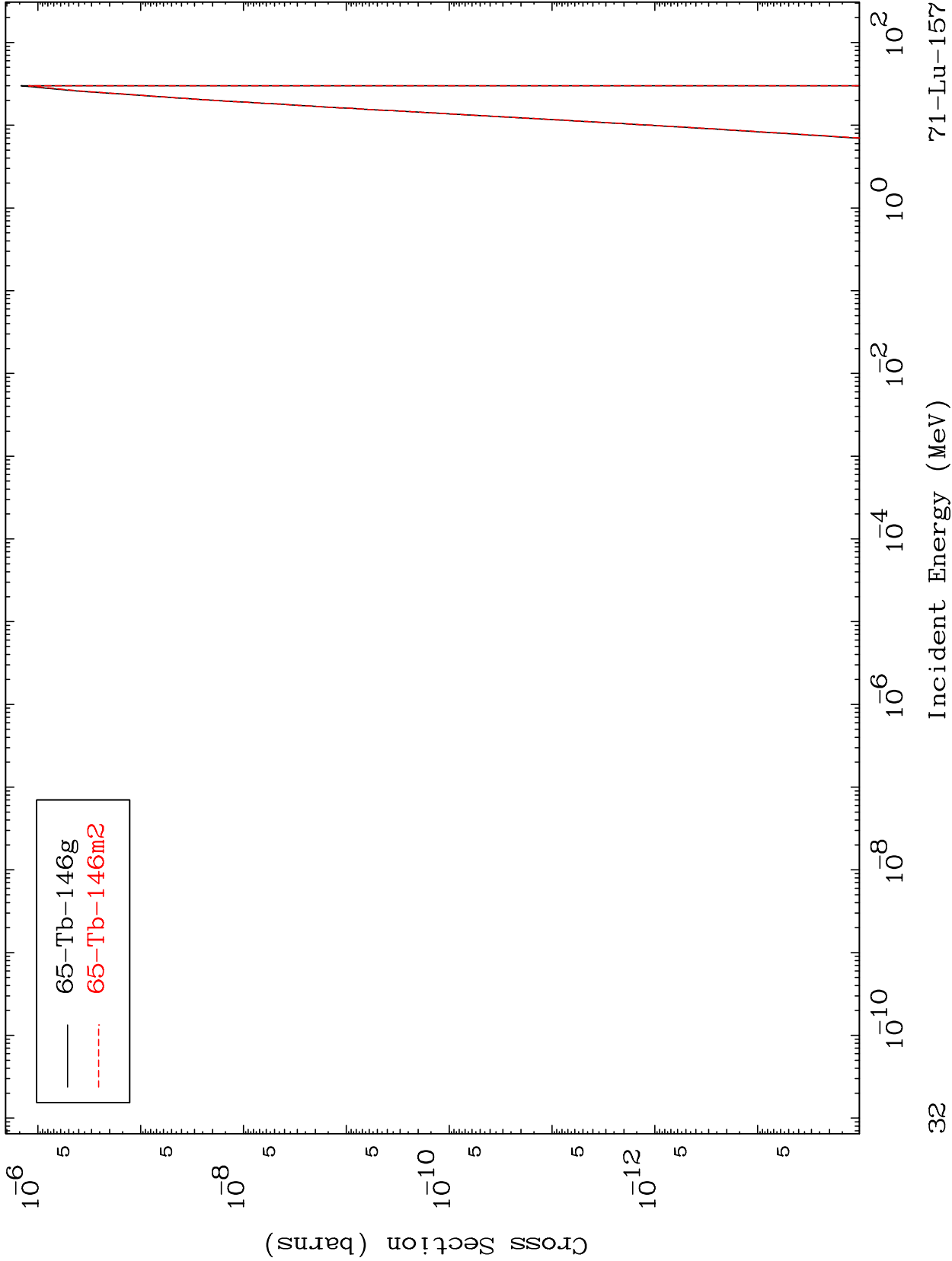


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(n, 3 α)

⁷¹Lu-157

Radionuclide Production Cross Section

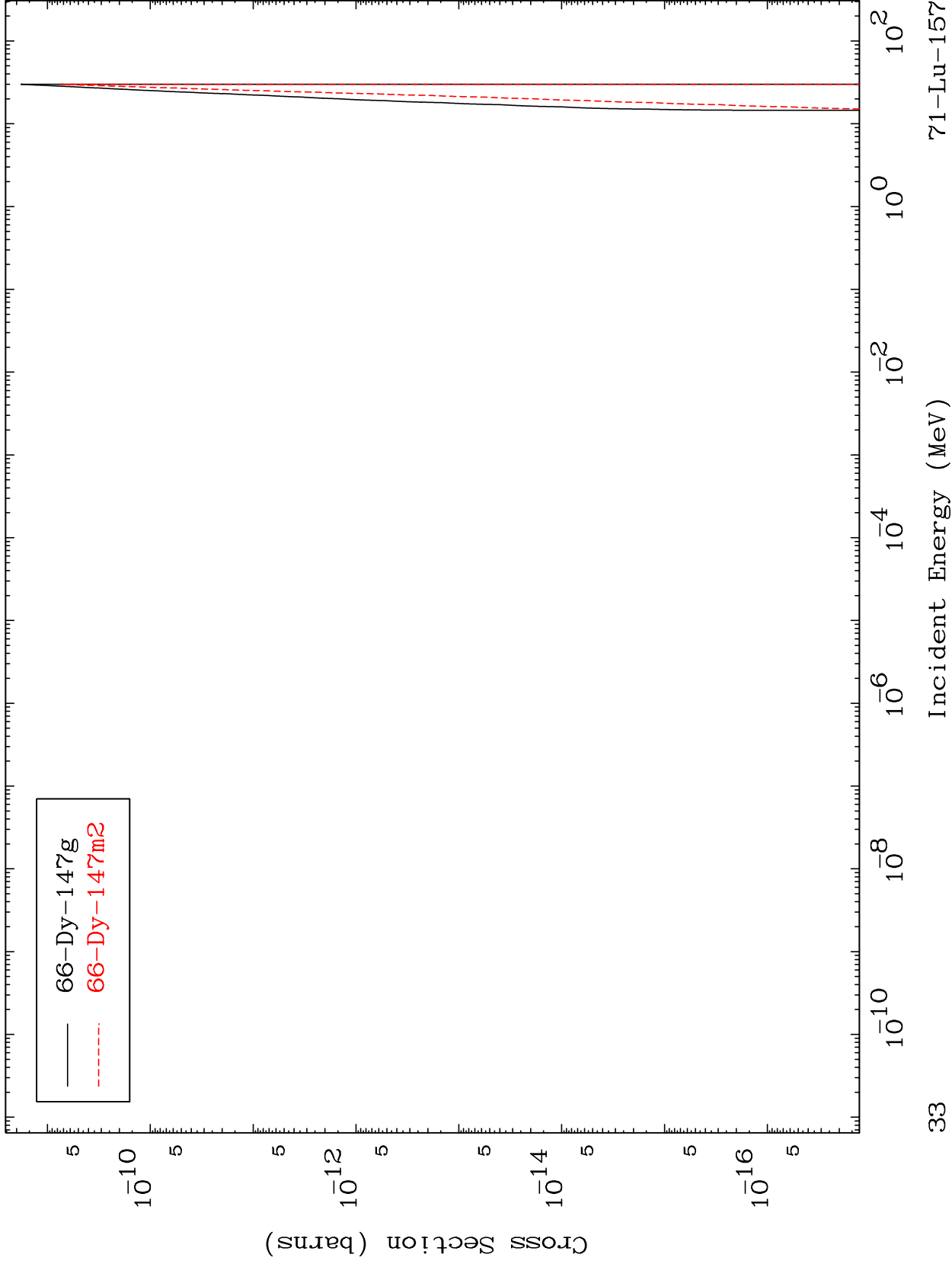


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(n,t) 2 α

71-Lu-157

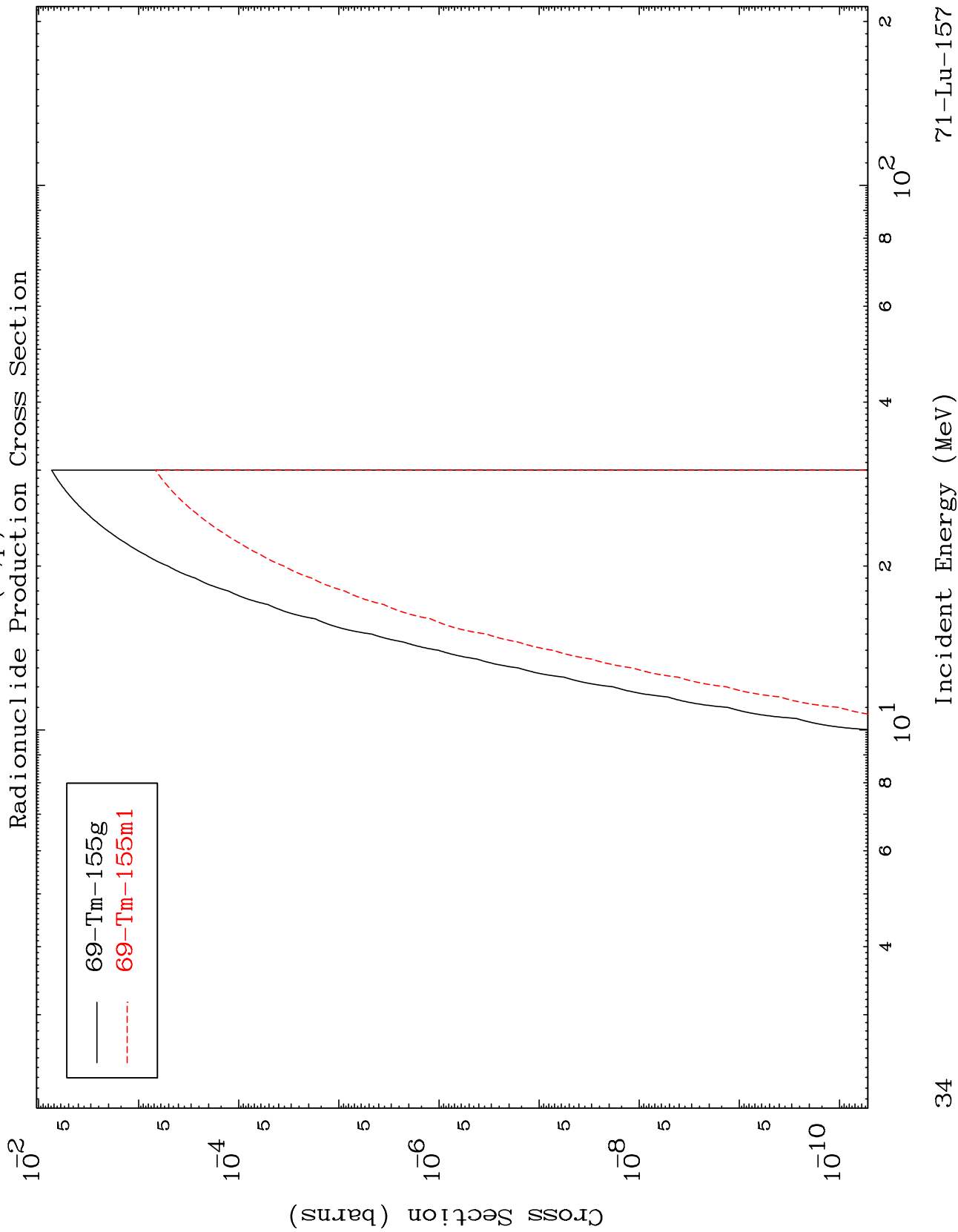
Radionuclide Production Cross Section



MAT 7072

(n,p) d

71-Lu-157

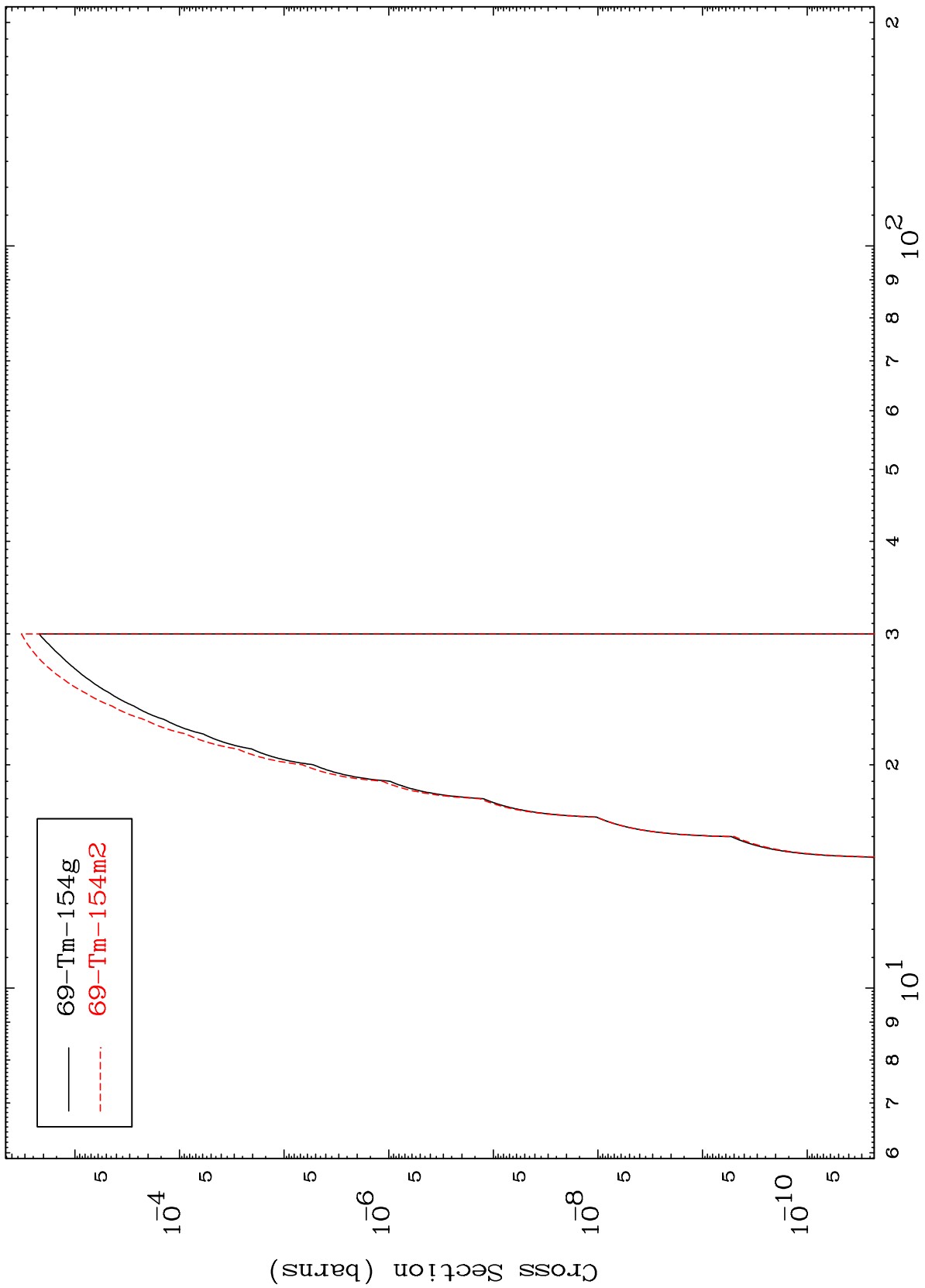


MAT 7072

(n,p) t

71-Lu-157

Radionuclide Production Cross Section



35

Incident Energy (MeV)

71-Lu-157