

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

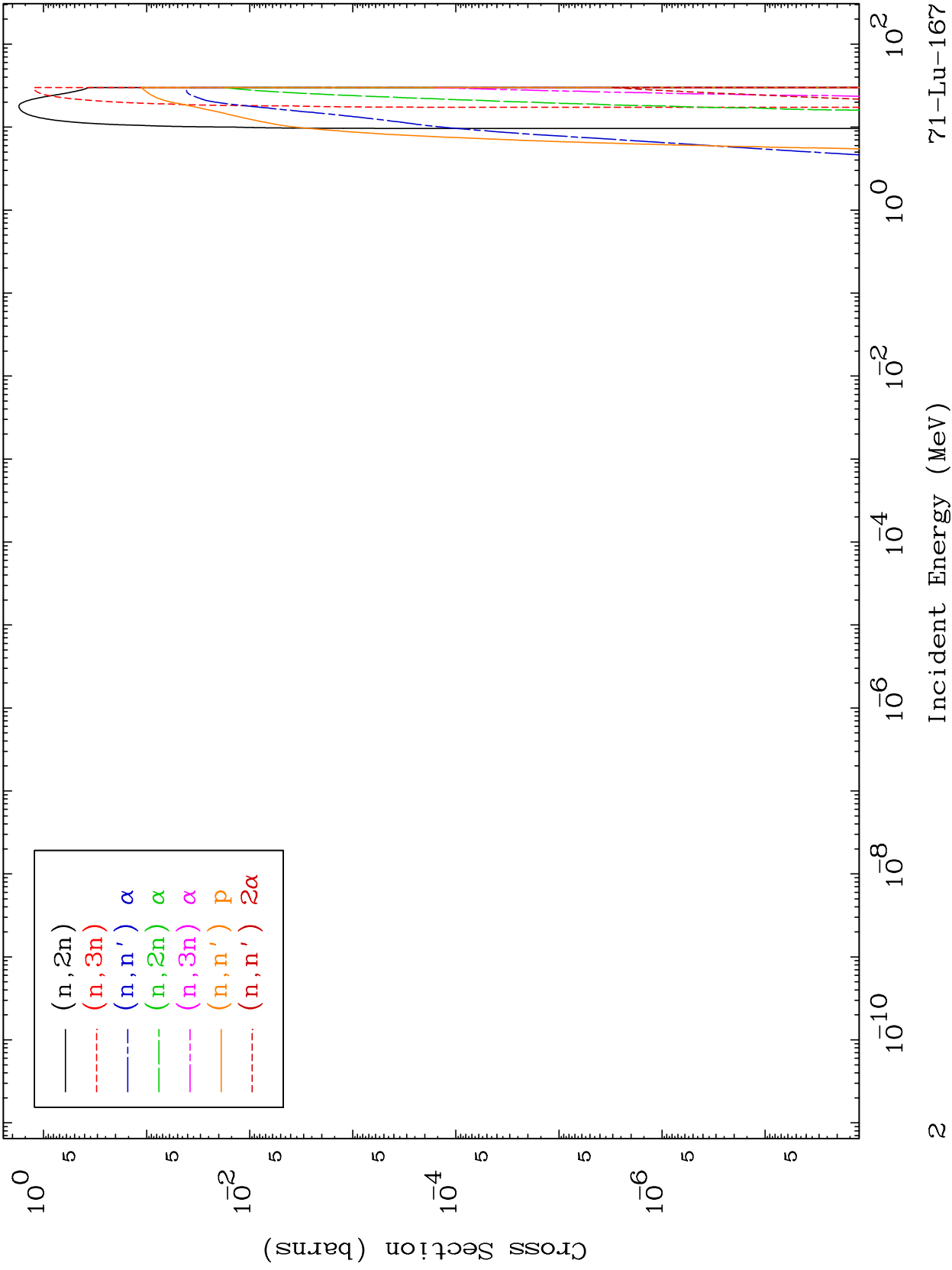
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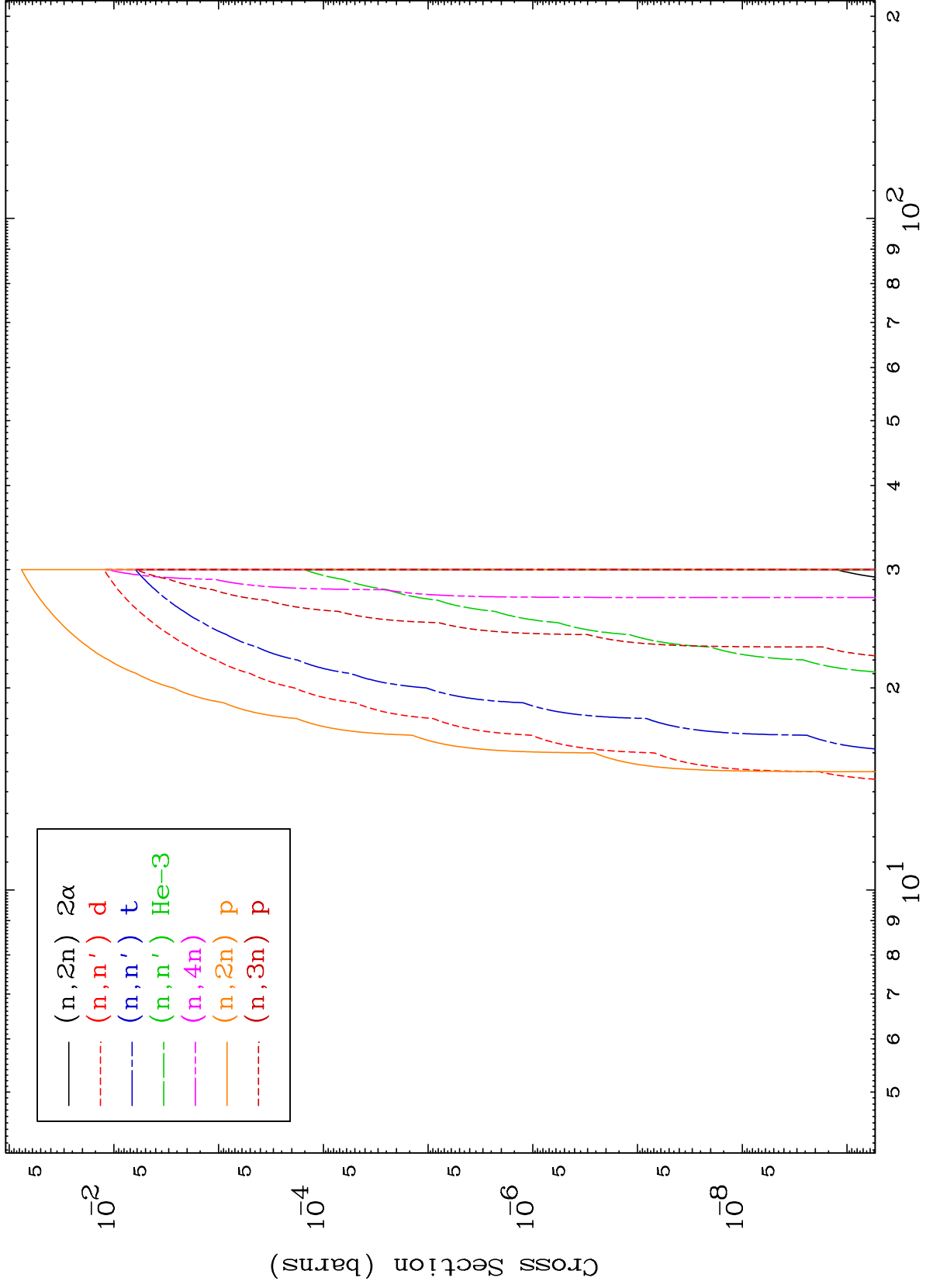
Press Mouse Button to Start

MAT 7101

Neutron Production
293 Kelvin Cross Sections

71-Lu-167

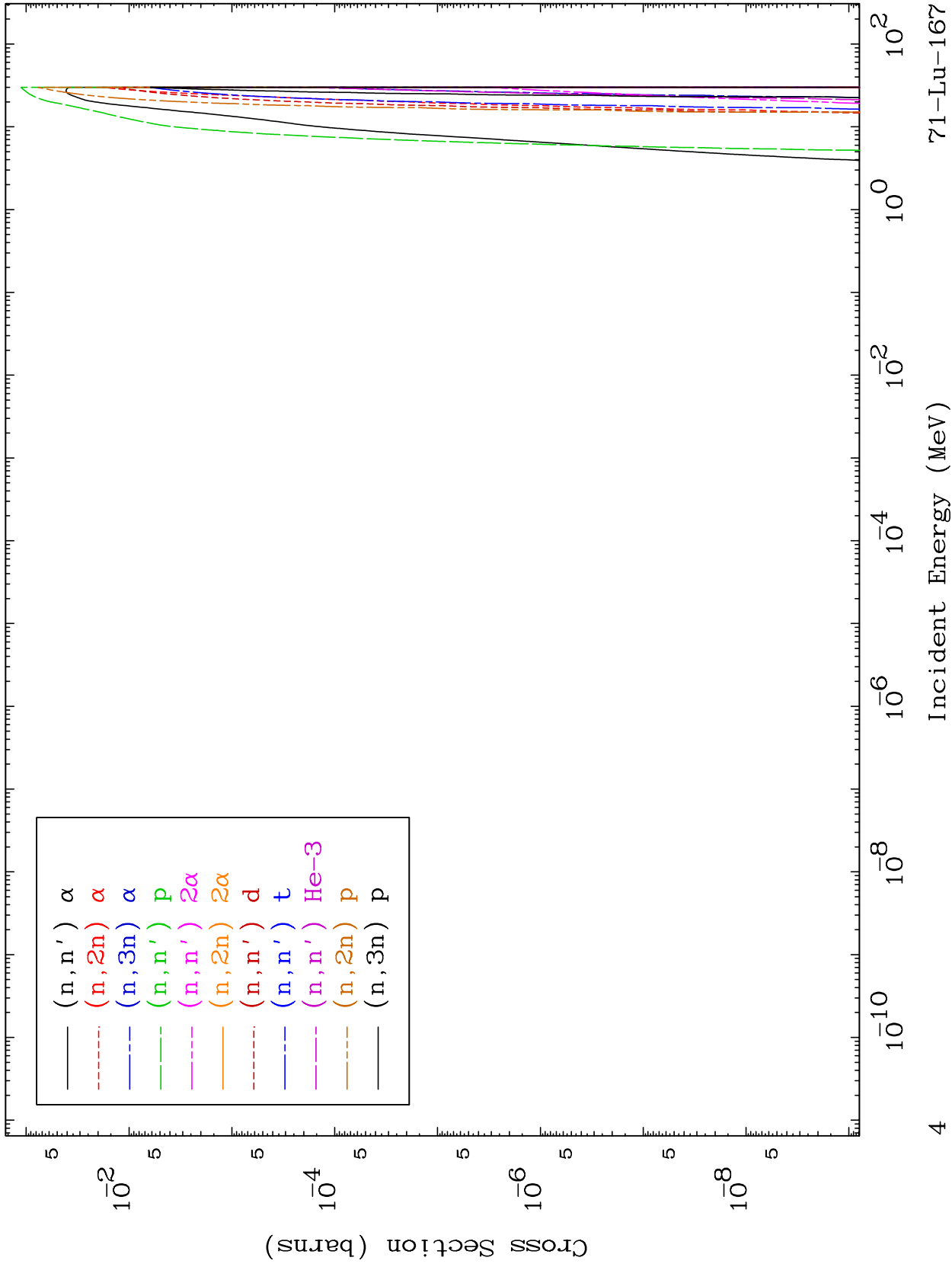




MAT 7101

Charged Particle
293 Kelvin Cross Sections

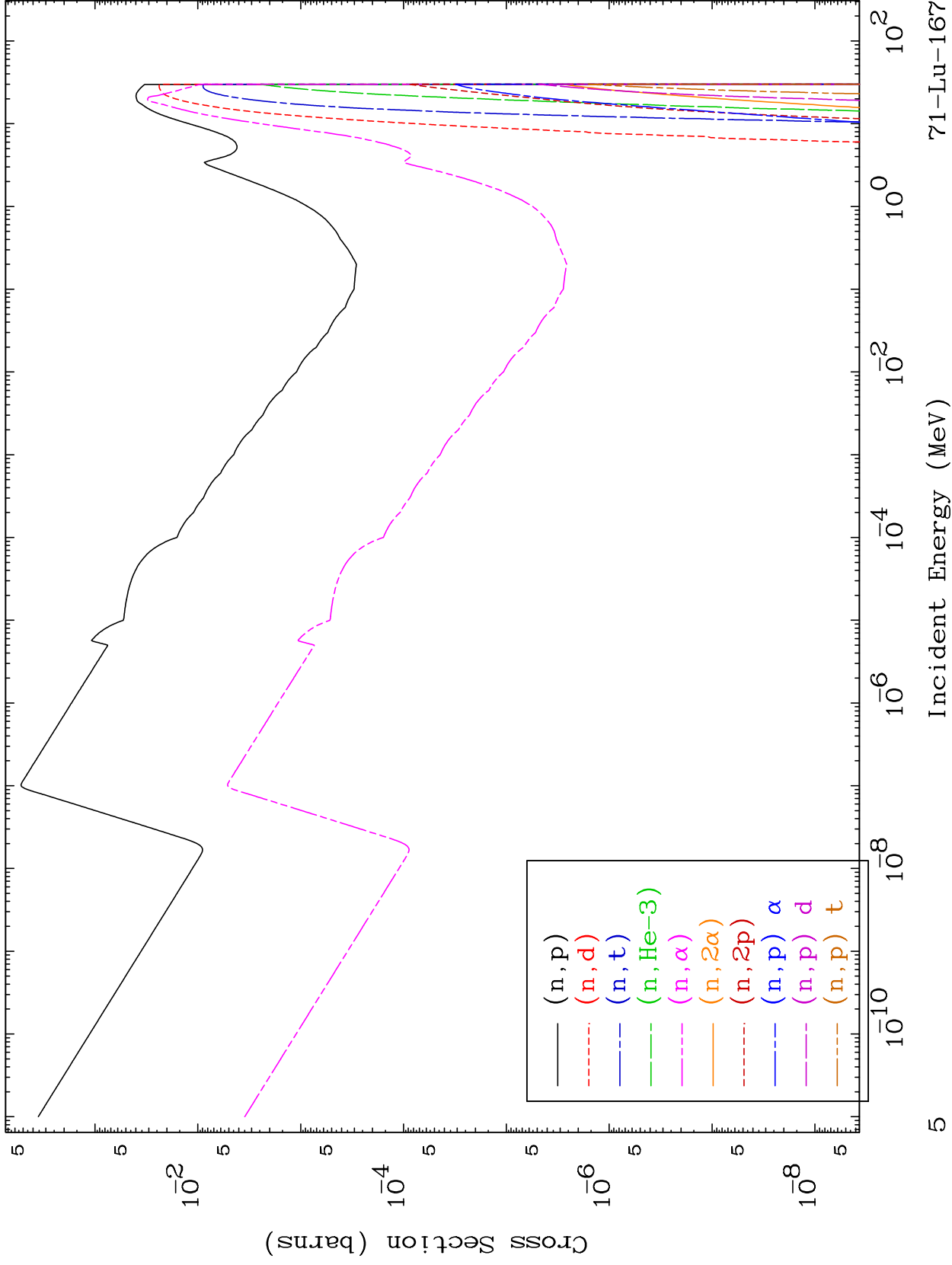
71-Lu-167



MAT 7101

Charged Particle
293 Kelvin Cross Sections

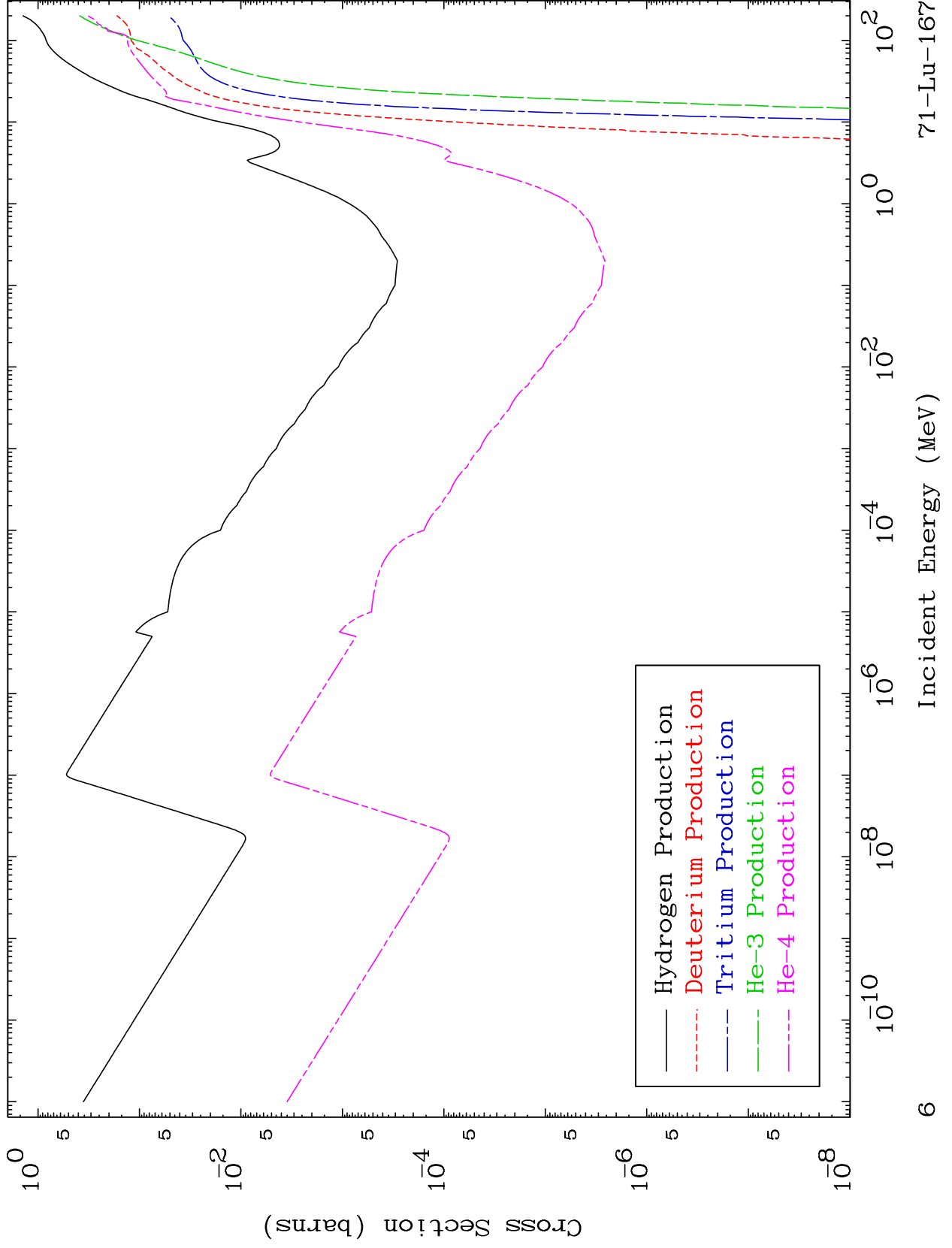
71-Lu-167

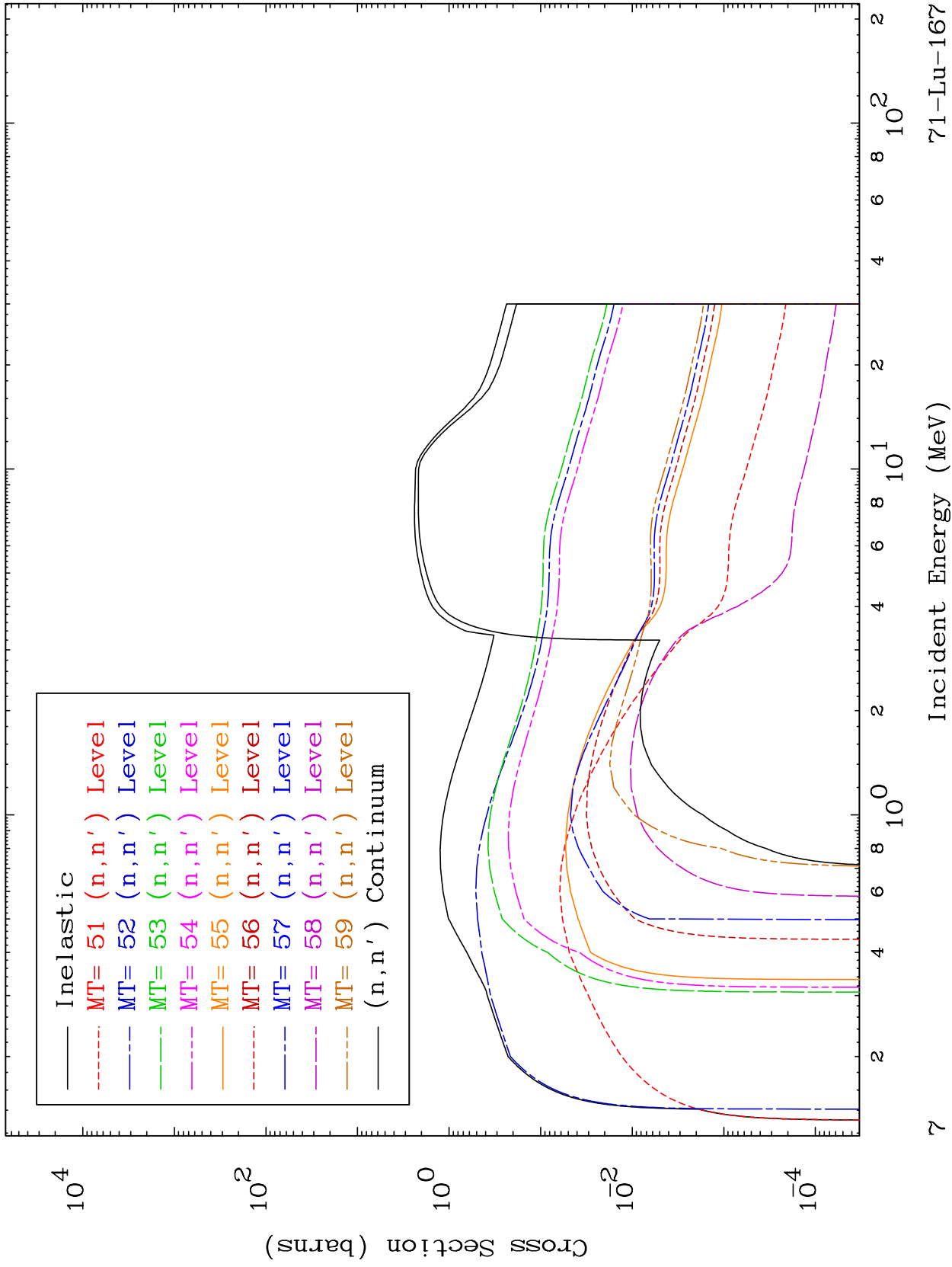


MAT 7101

Particle Production
293 Kelvin Cross Sections

71-Lu-167

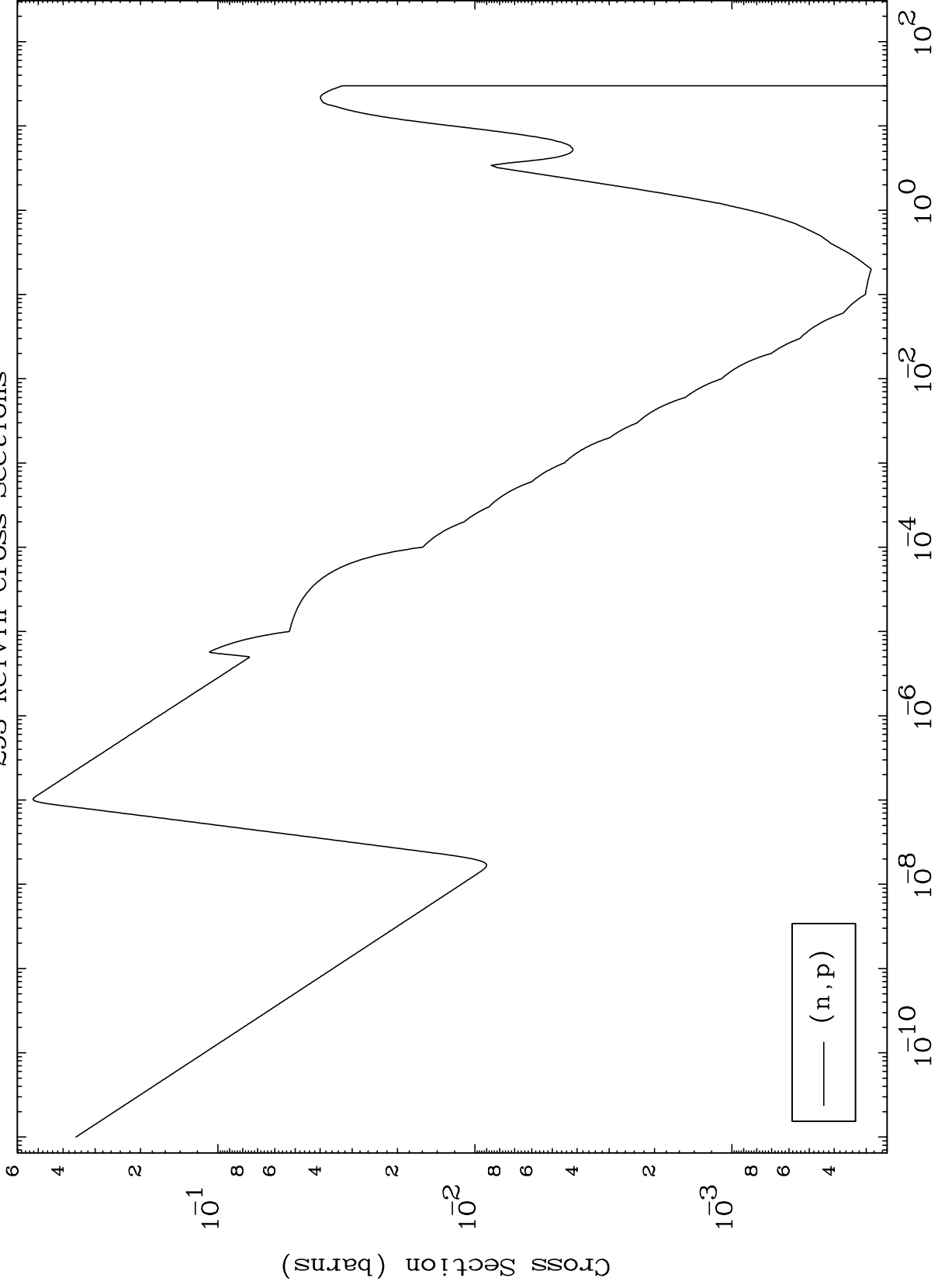




MAT 7101

(n,p) Levels
293 Kelvin Cross Sections

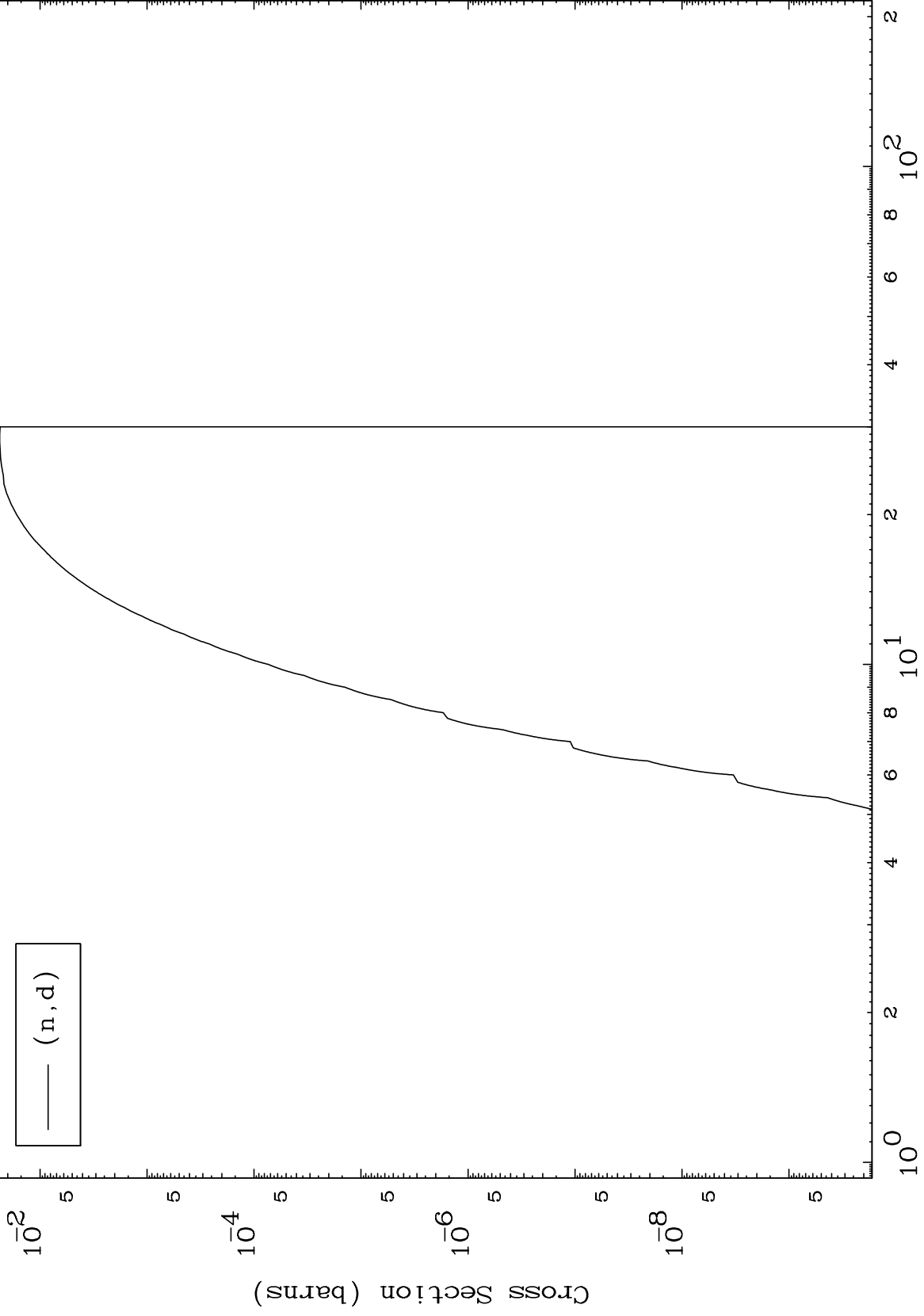
71-Lu-167



MAT 7101

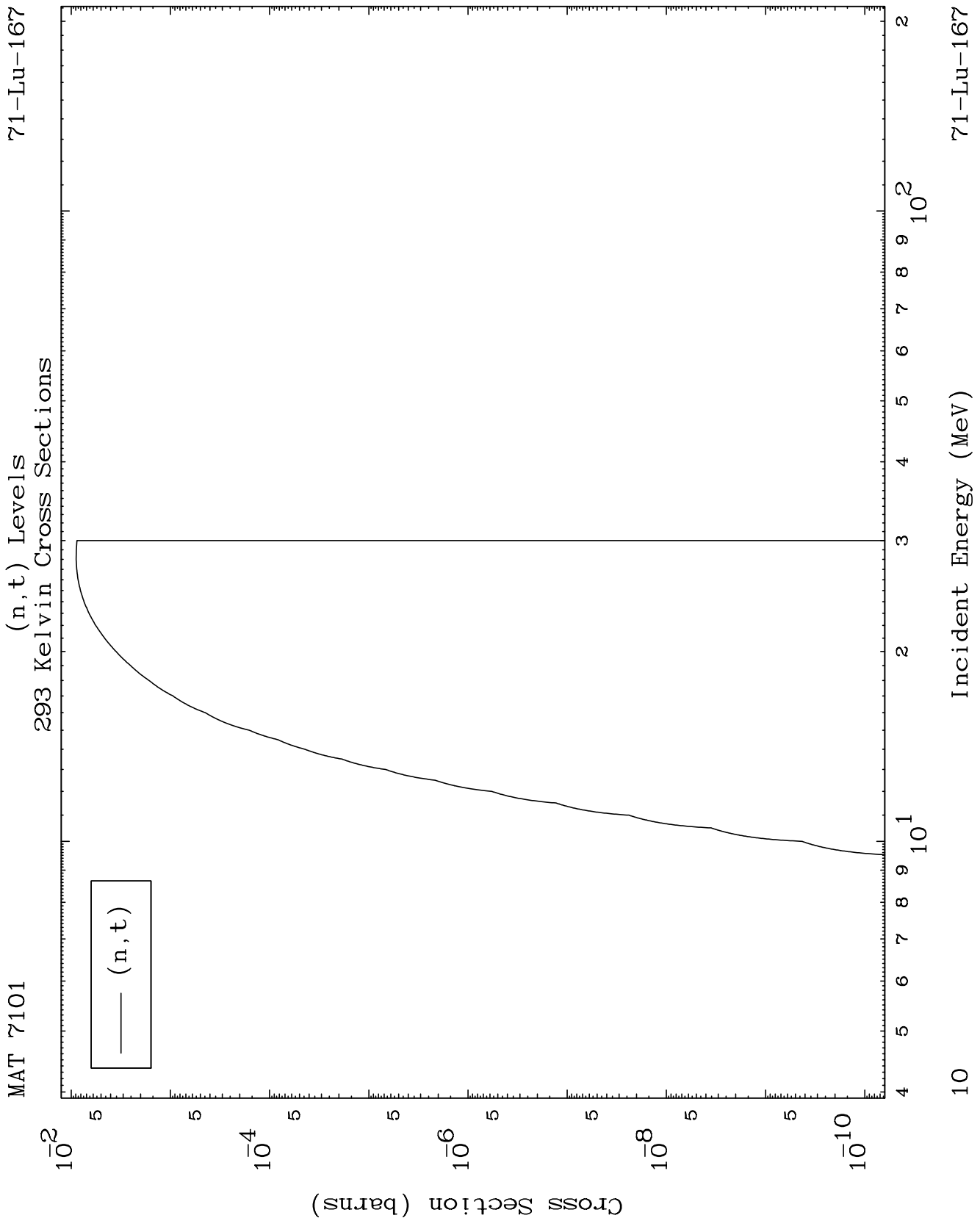
(n,d) Levels
293 Kelvin Cross Sections

71-Lu-167



Incident Energy (MeV)

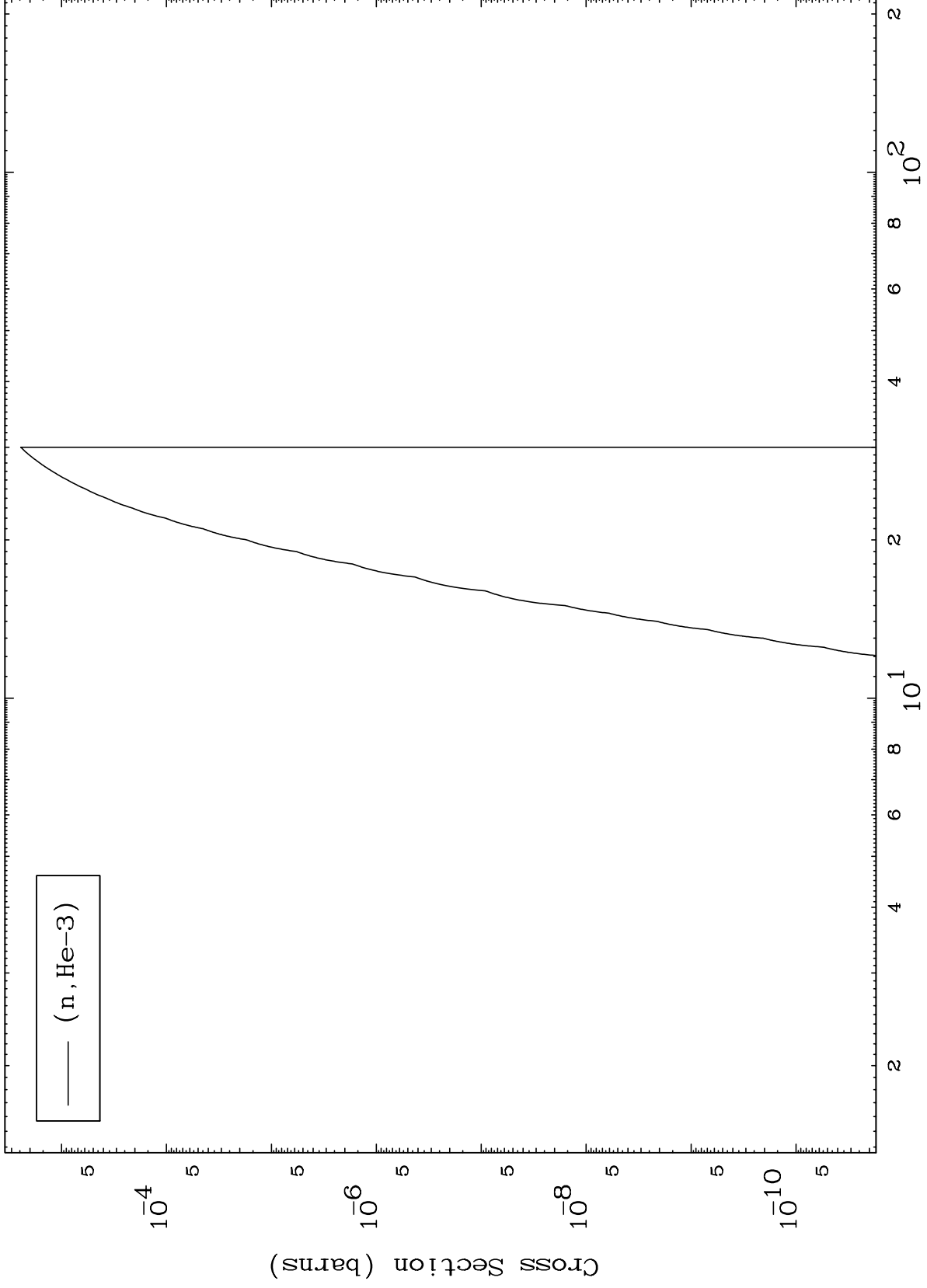
71-Lu-167



MAT 7101

(n,He3) Levels
293 Kelvin Cross Sections

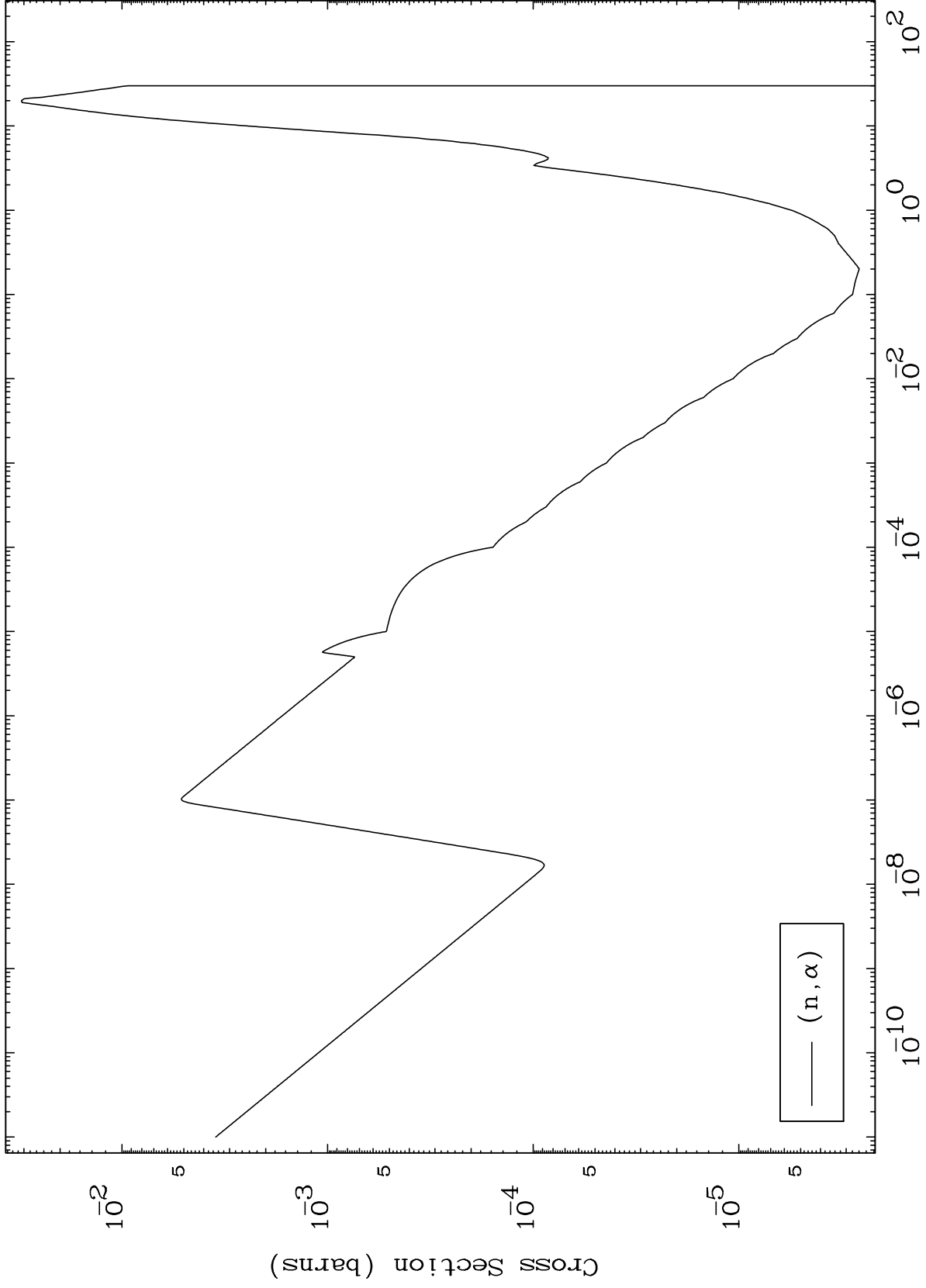
71-Lu-167



MAT 7101

(n,α) Levels
293 Kelvin Cross Sections

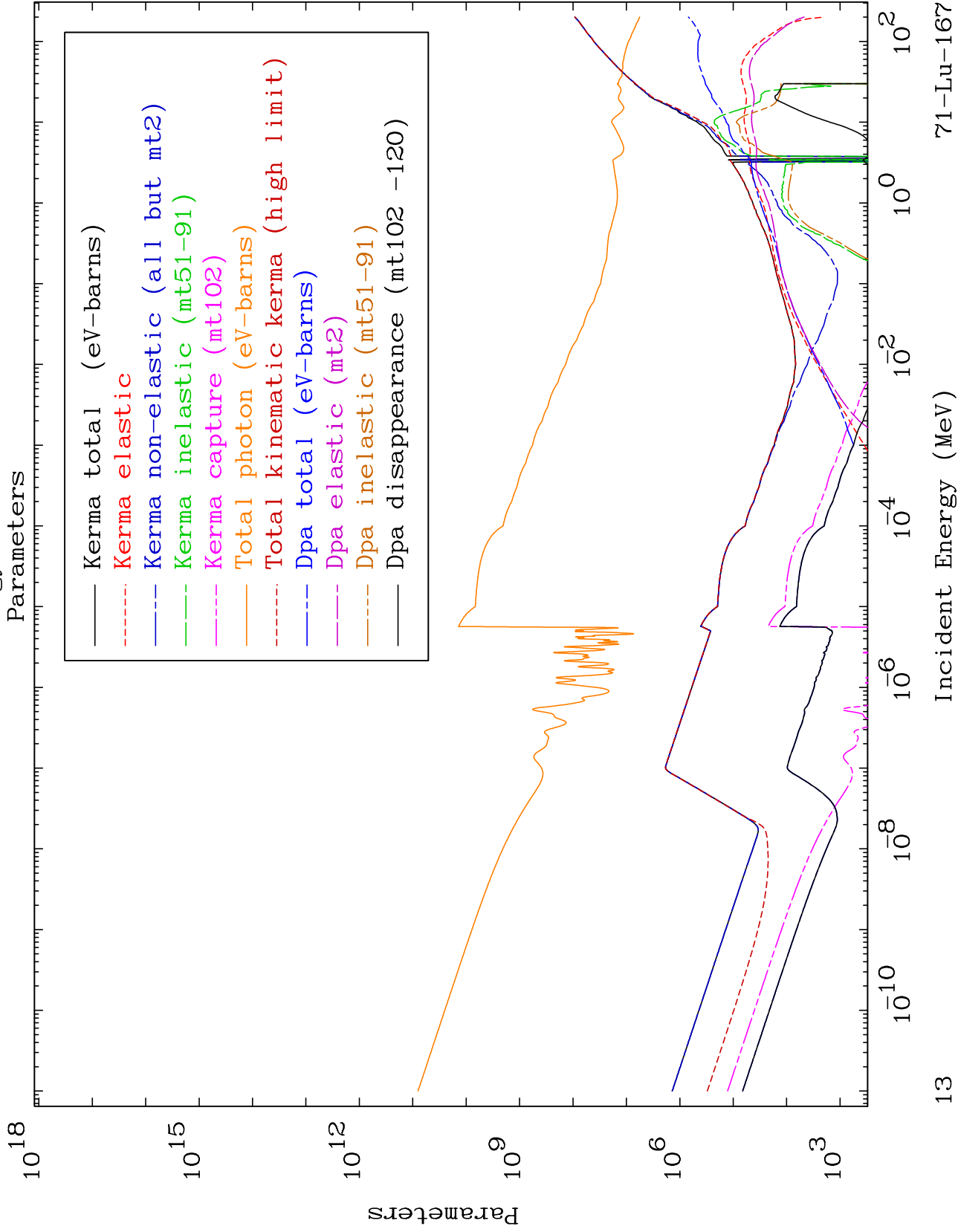
71-Lu-167

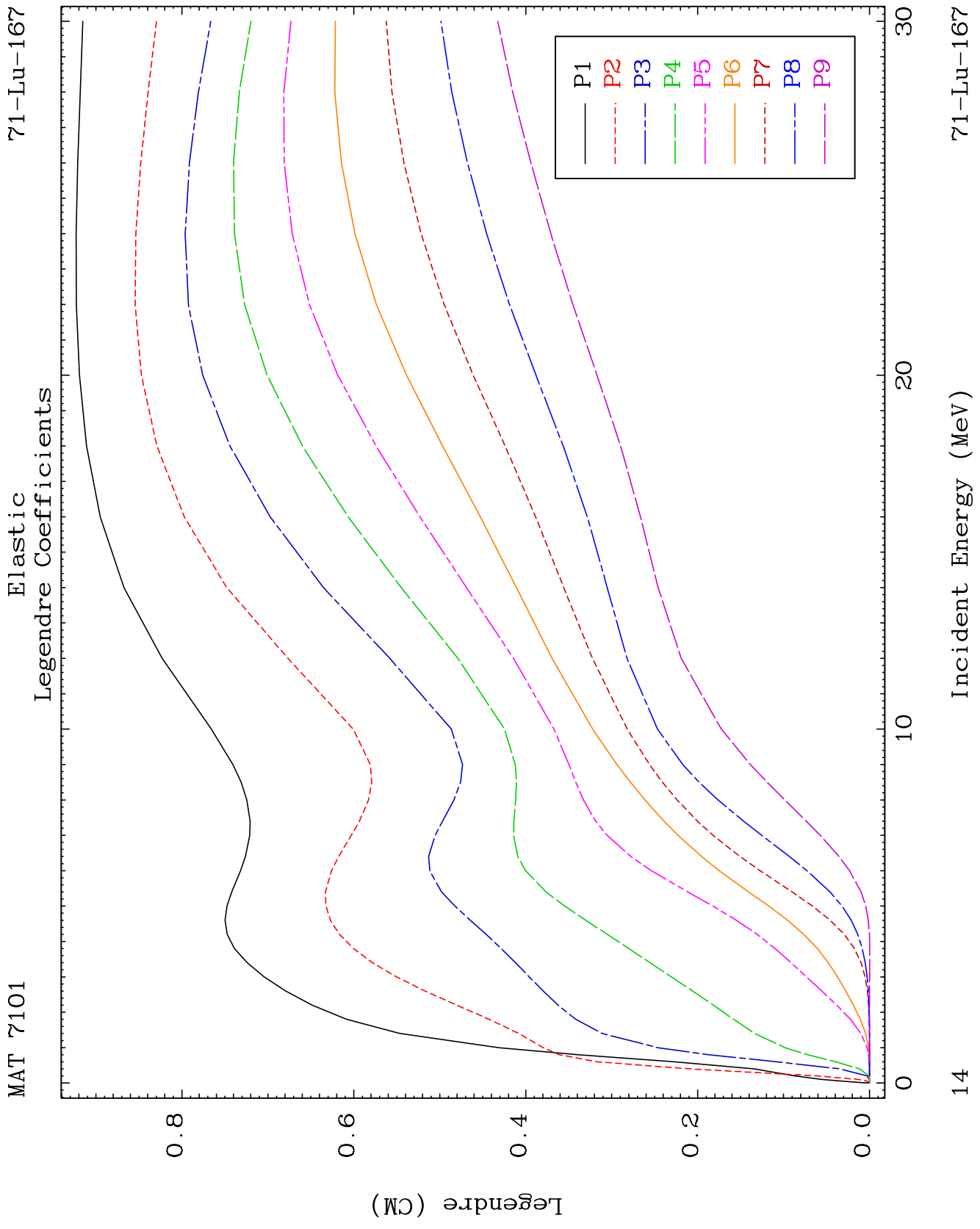


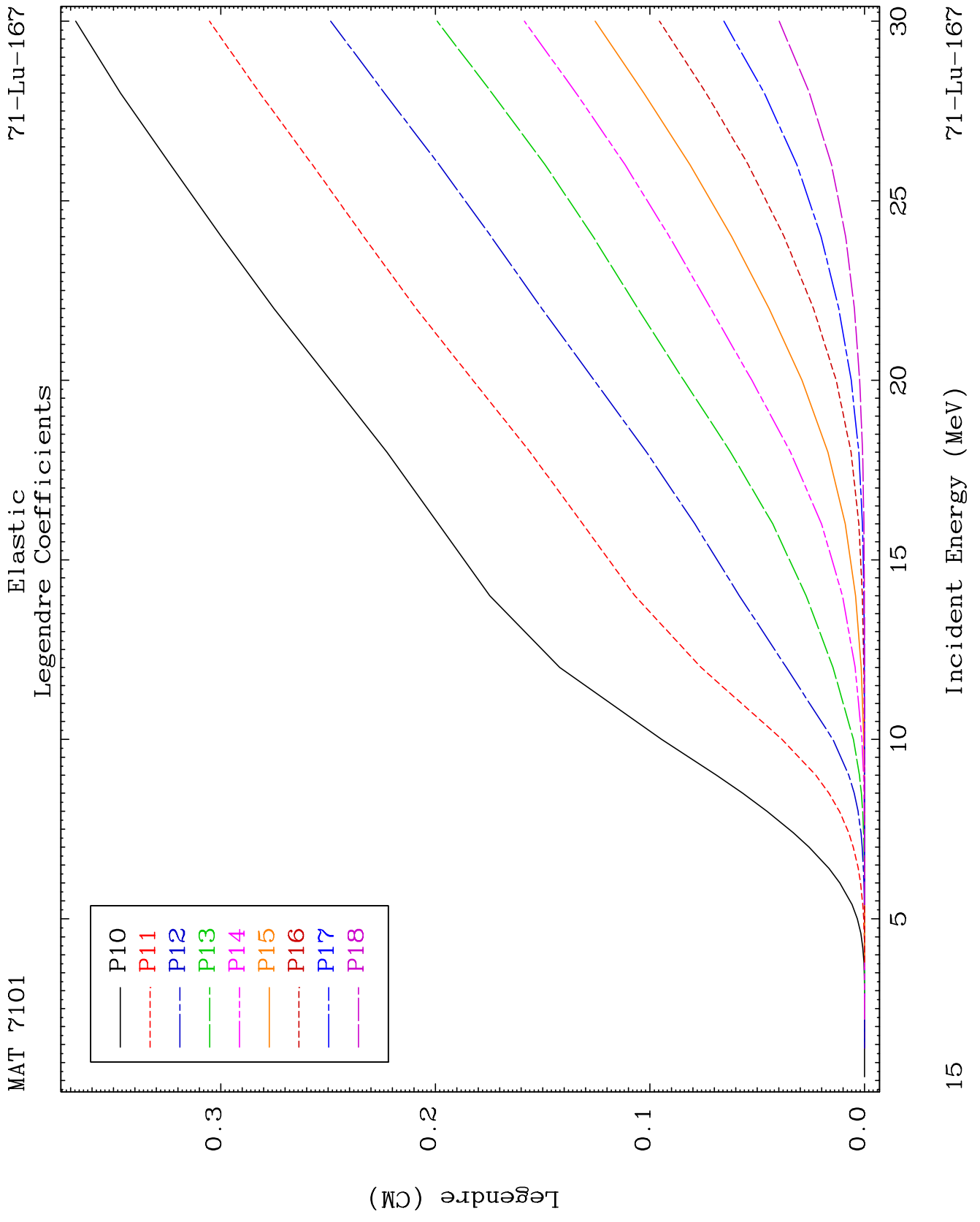
12

Incident Energy (MeV)

71-Lu-167



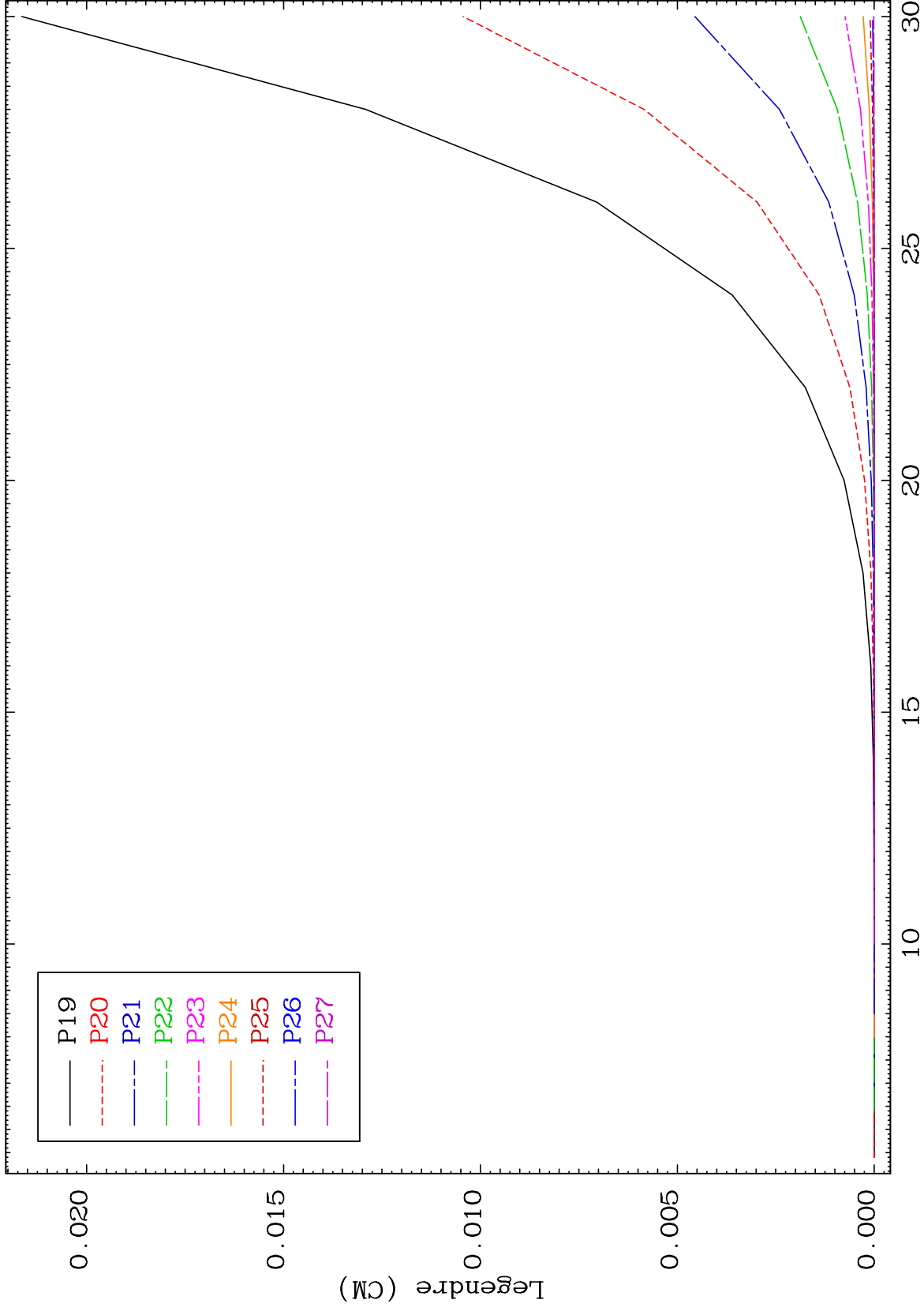




MAT 7101

Elastic Legendre Coefficients

71-Lu-167



16

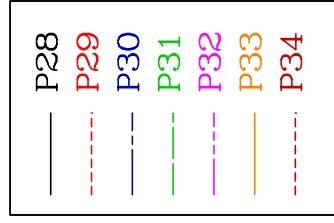
Incident Energy (MeV)

71-Lu-167

MAT 7101

Elastic Legendre Coefficients

71-Lu-167



$\times 10^{-6}$

Legendre (CM)

10

15

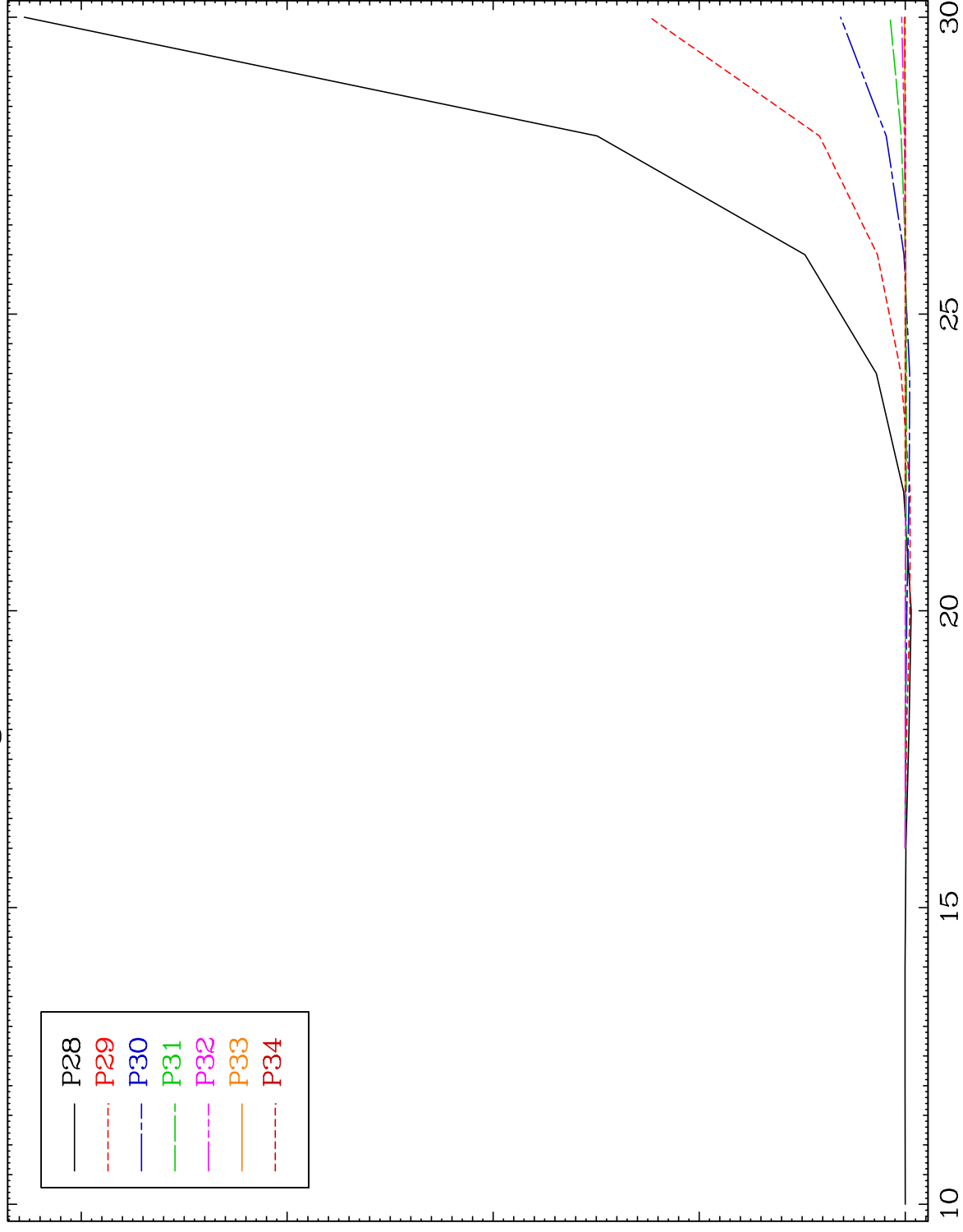
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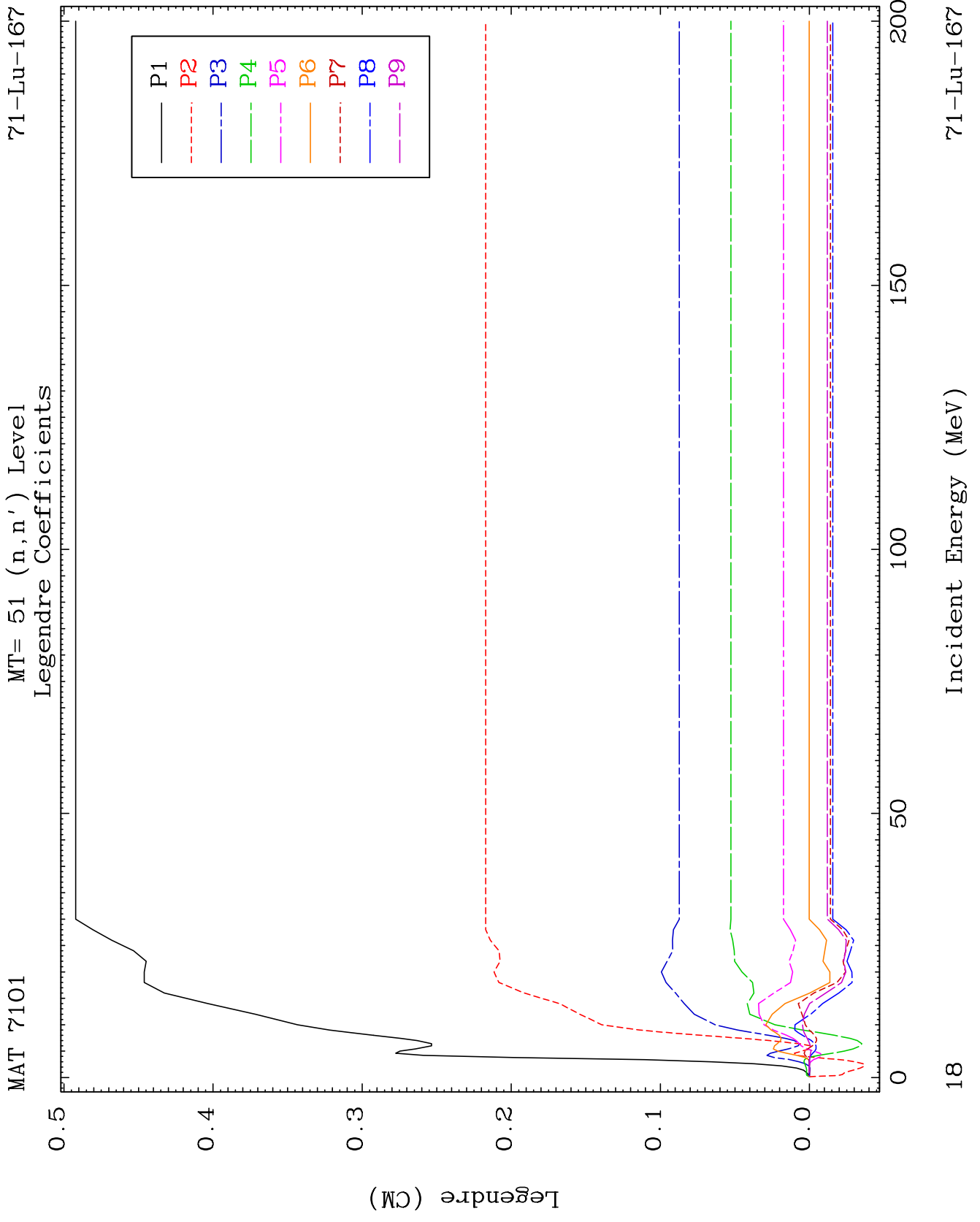
25

30

Incident Energy (MeV)

71-Lu-167

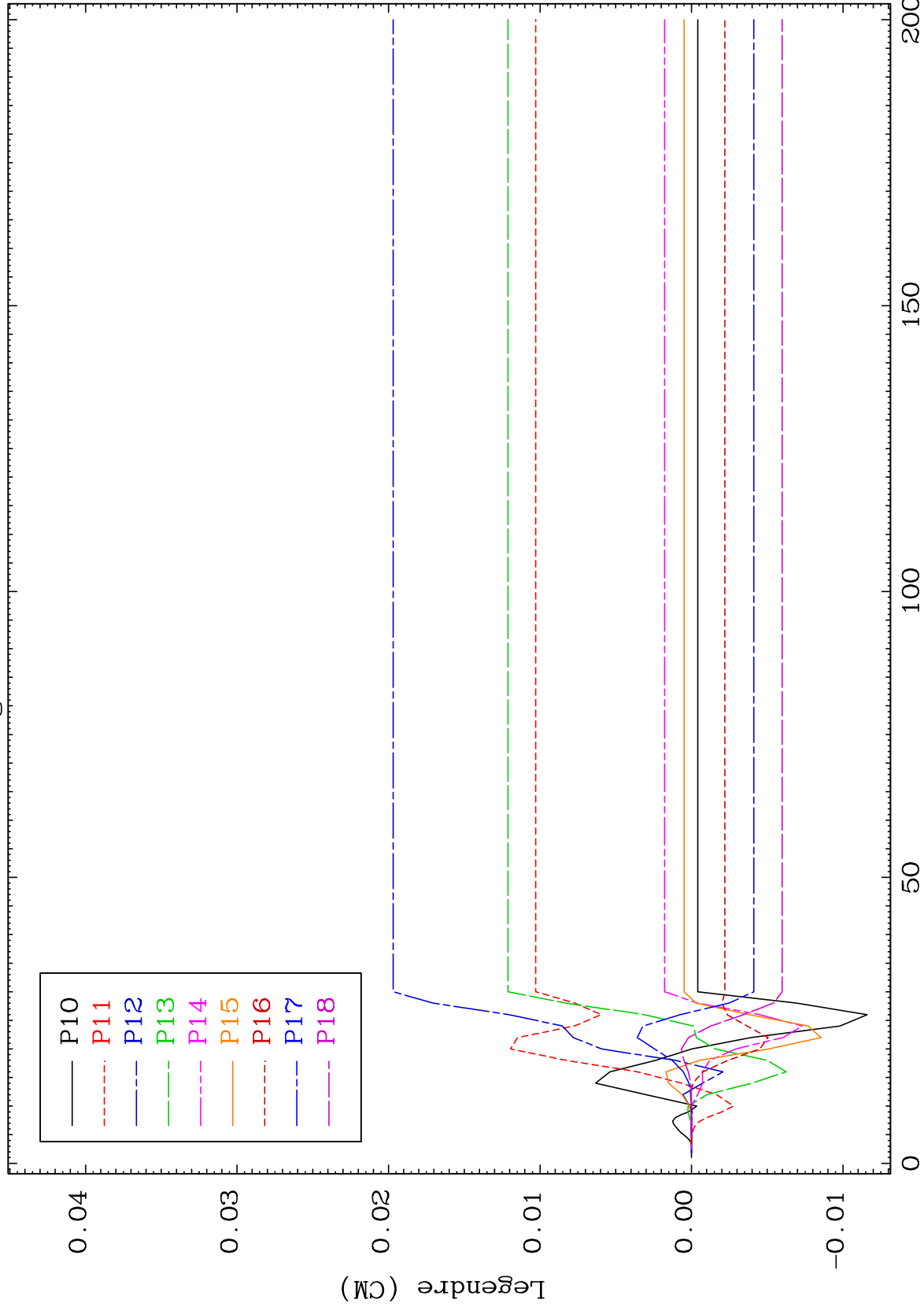




MAT 7101

MT= 51 (n,n') Level
Legendre Coefficients

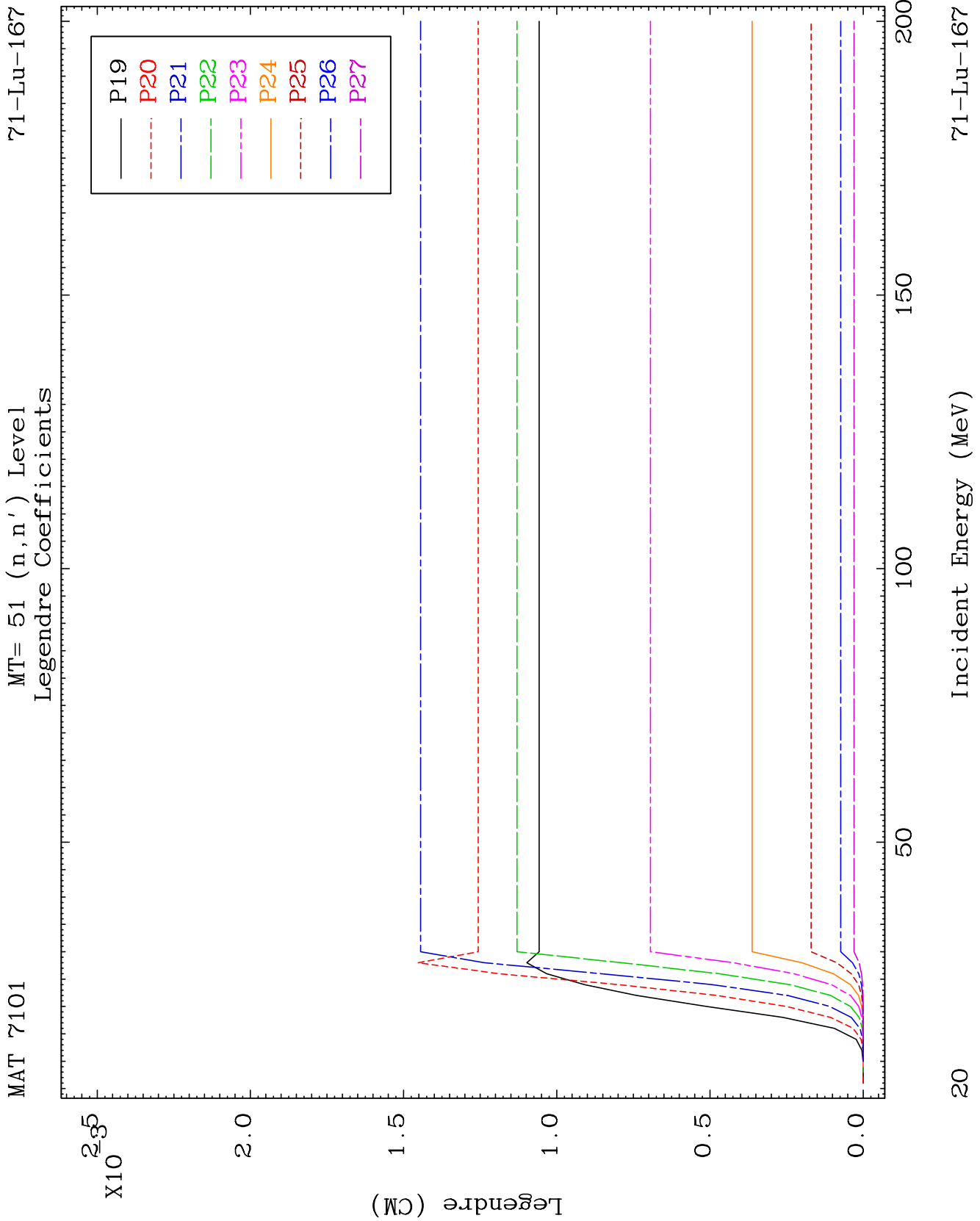
71-Lu-167



19

Incident Energy (MeV)

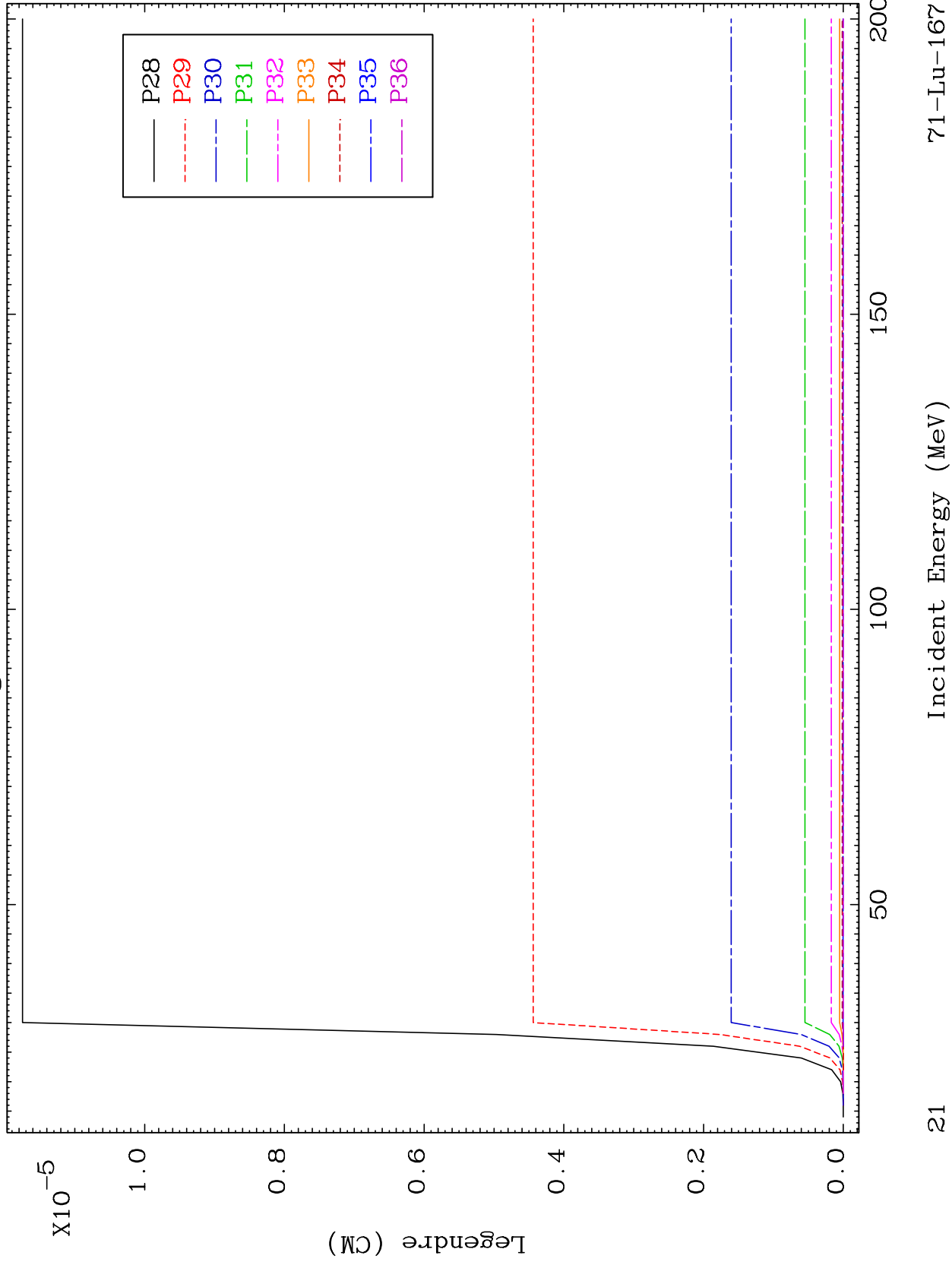
71-Lu-167



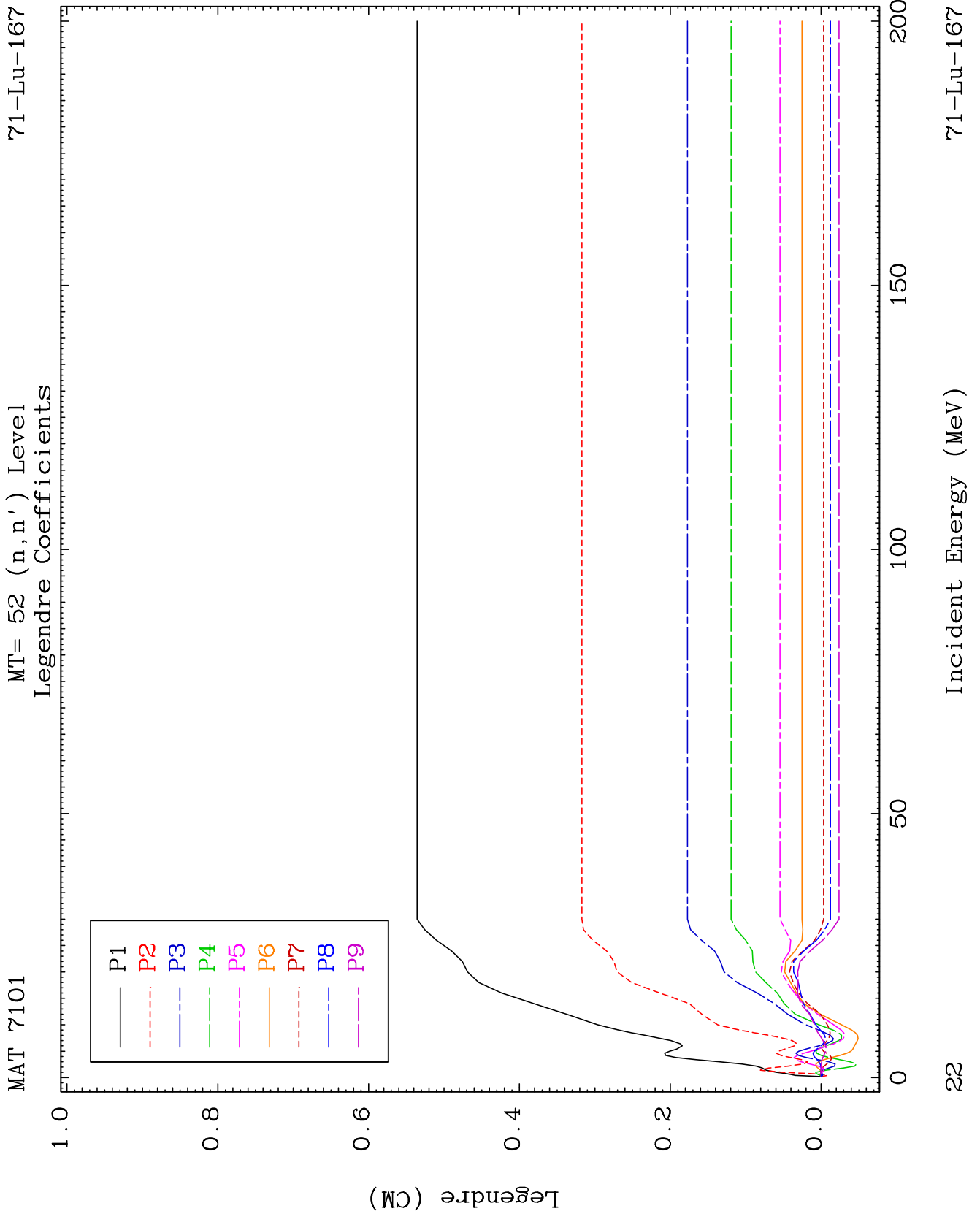
MAT 7101

MT= 51 (n,n') Level
Legendre Coefficients

71-Lu-167



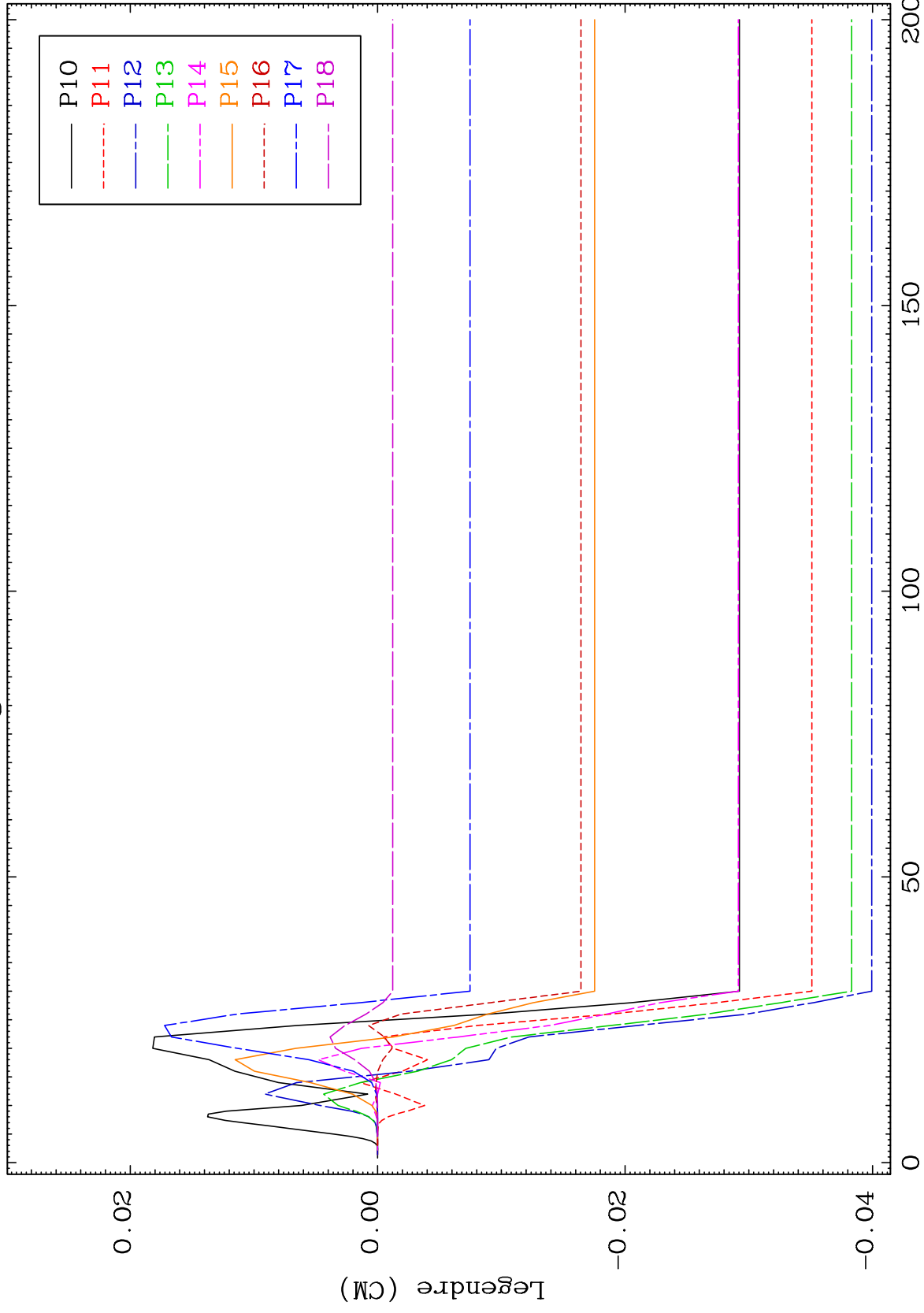
21



MAT 7101

MT= 52 (n,n') Level
Legendre Coefficients

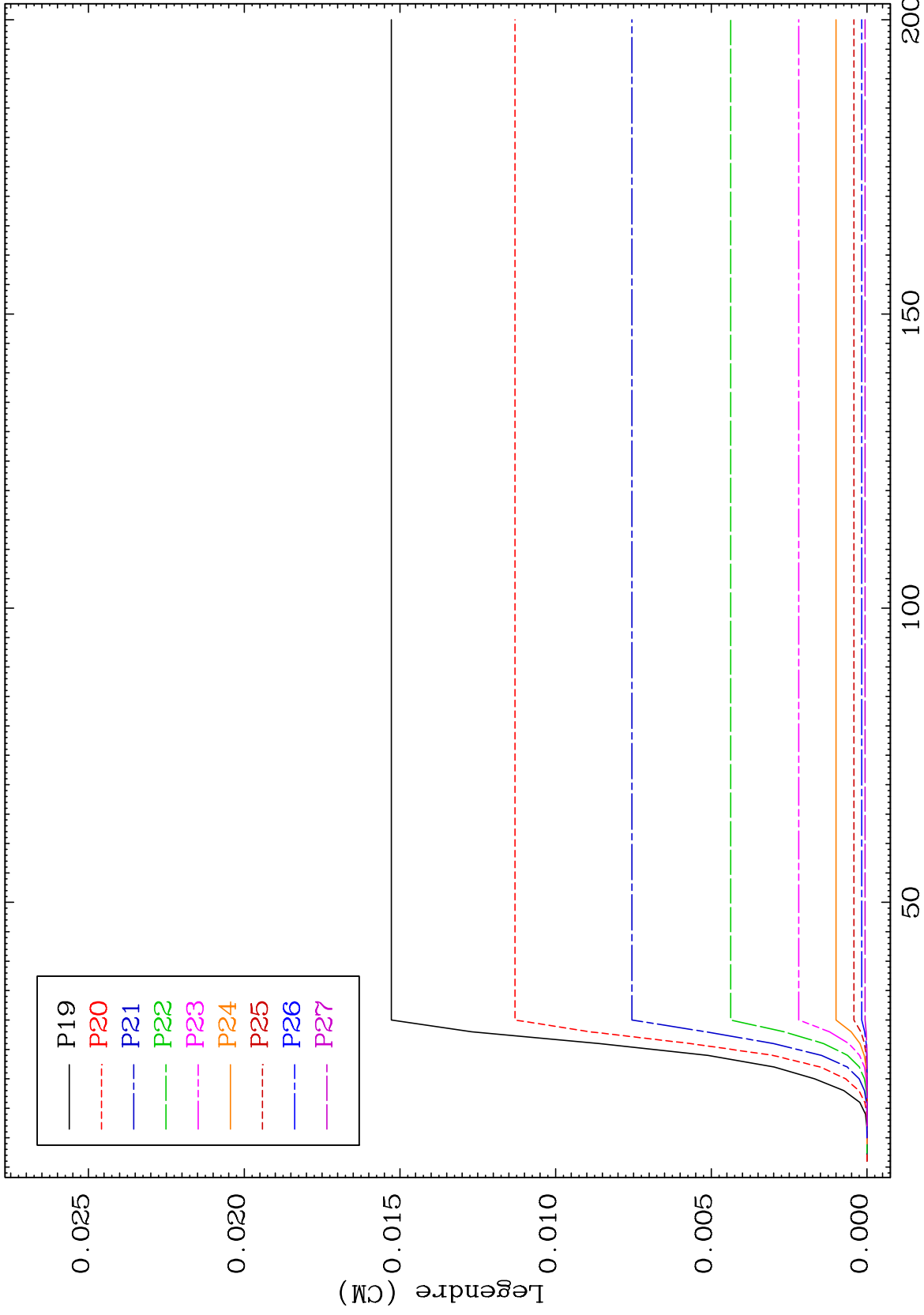
71-Lu-167



23

Incident Energy (MeV)

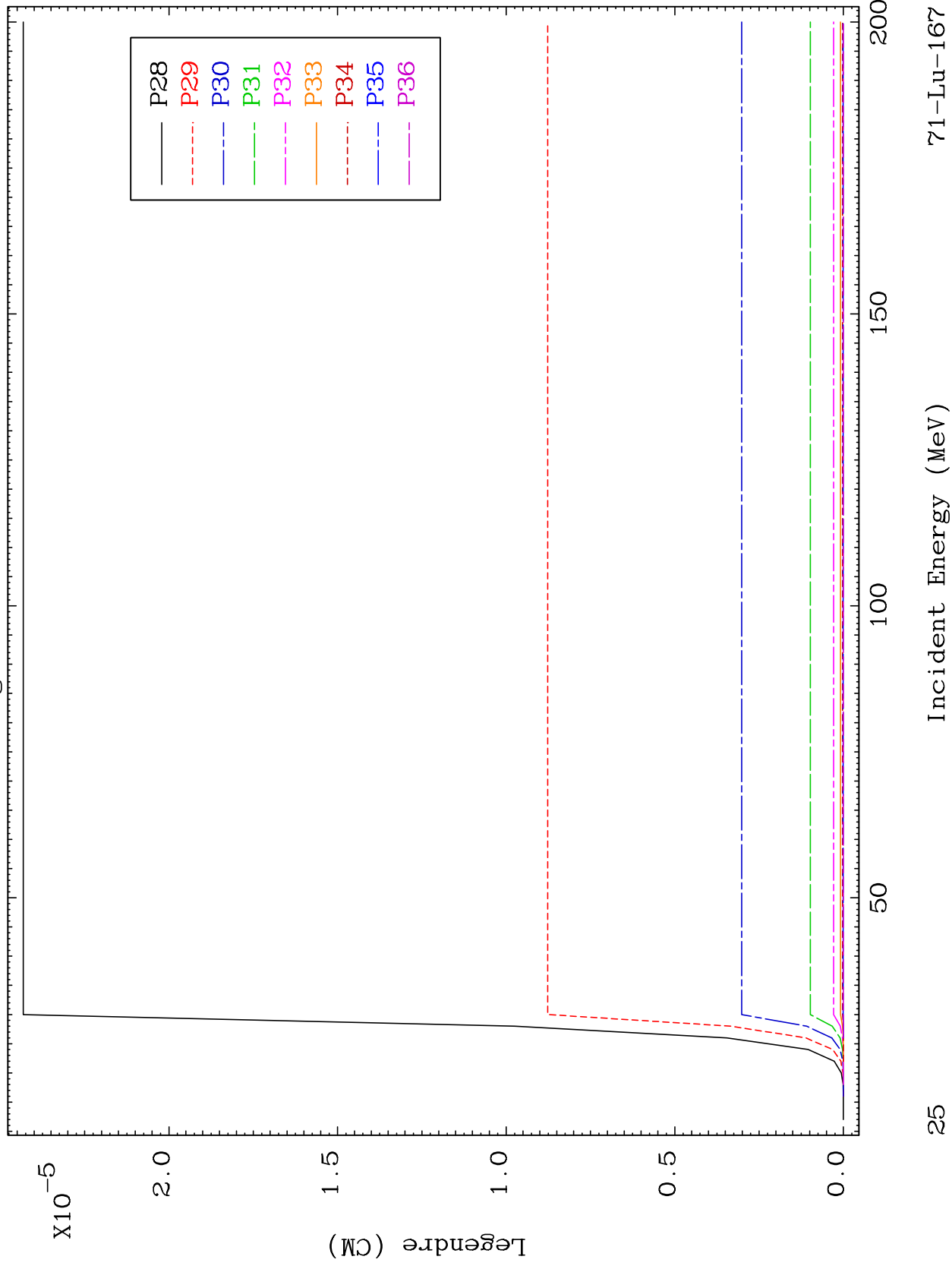
71-Lu-167



MAT 7101

MT= 52 (n,n') Level
Legendre Coefficients

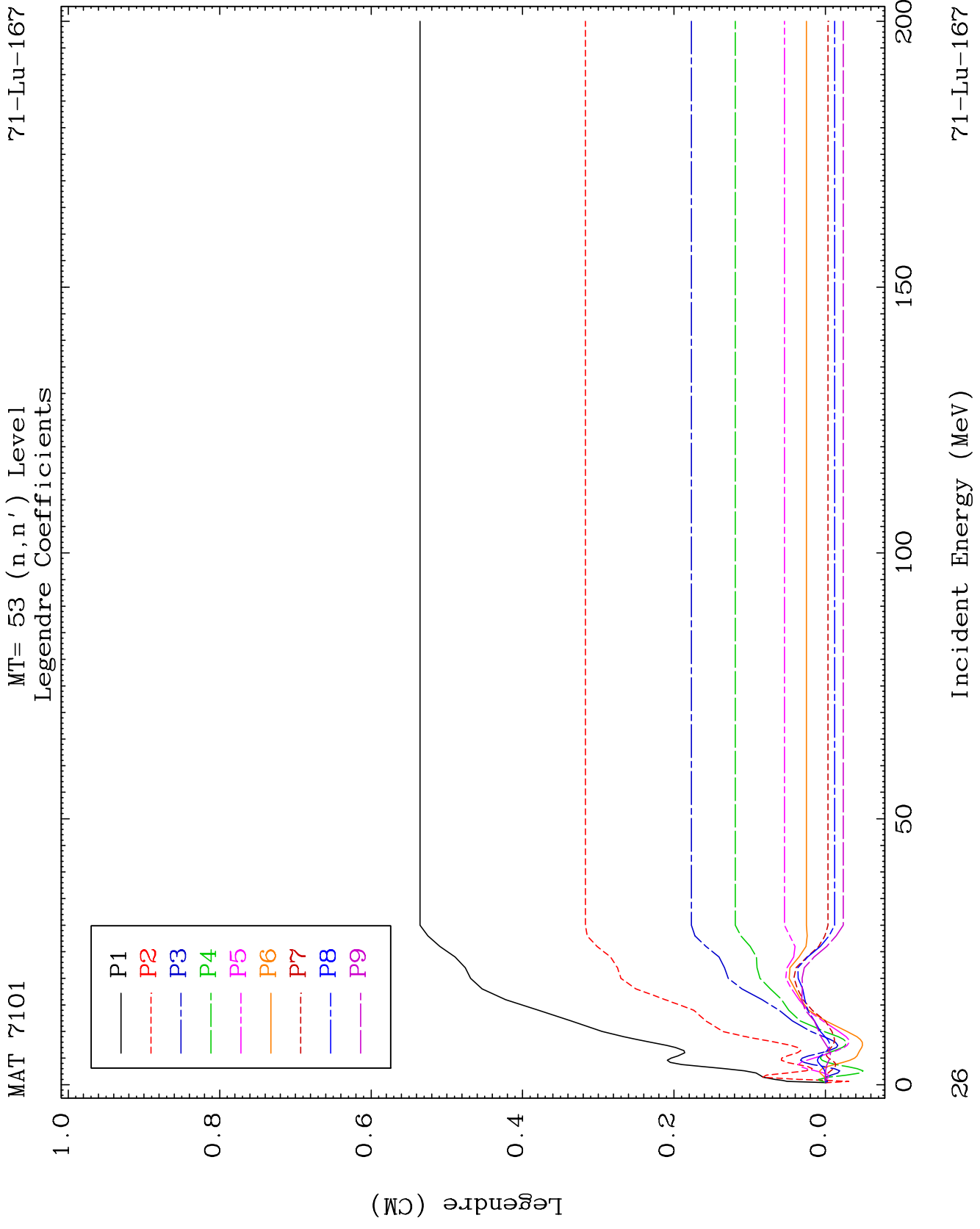
71-Lu-167



25

Incident Energy (MeV)

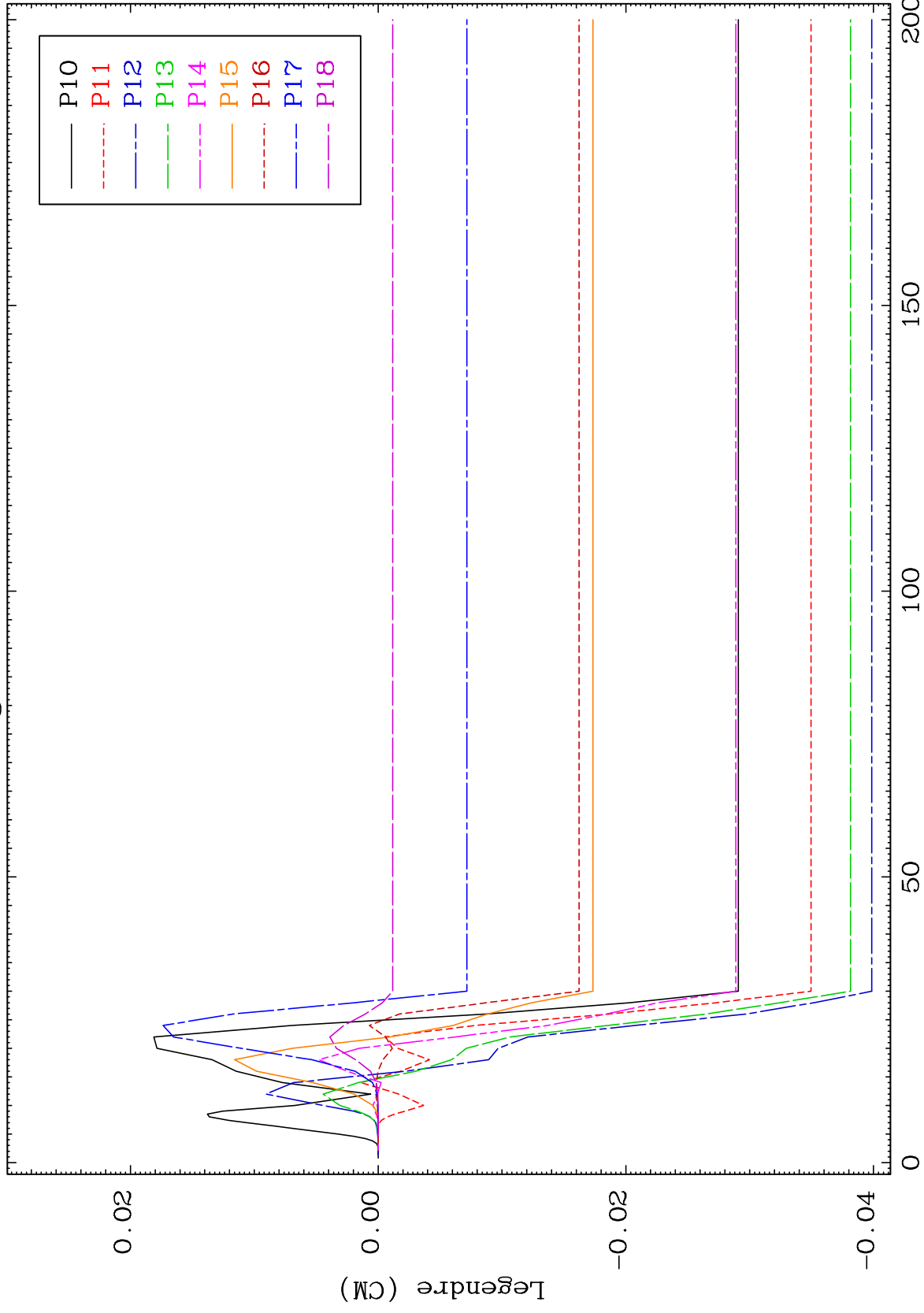
71-Lu-167



MAT 7101

MT= 53 (n,n') Level
Legendre Coefficients

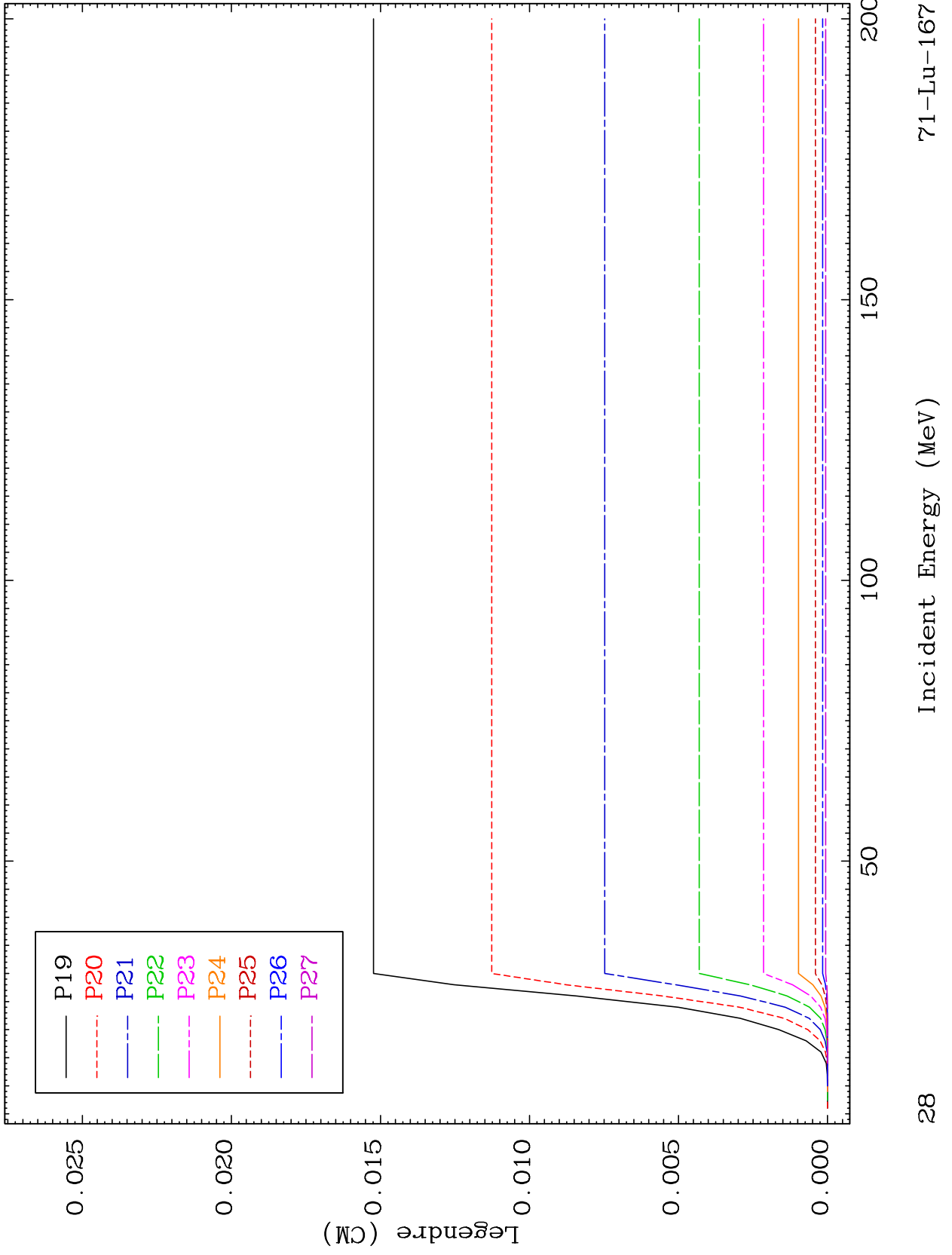
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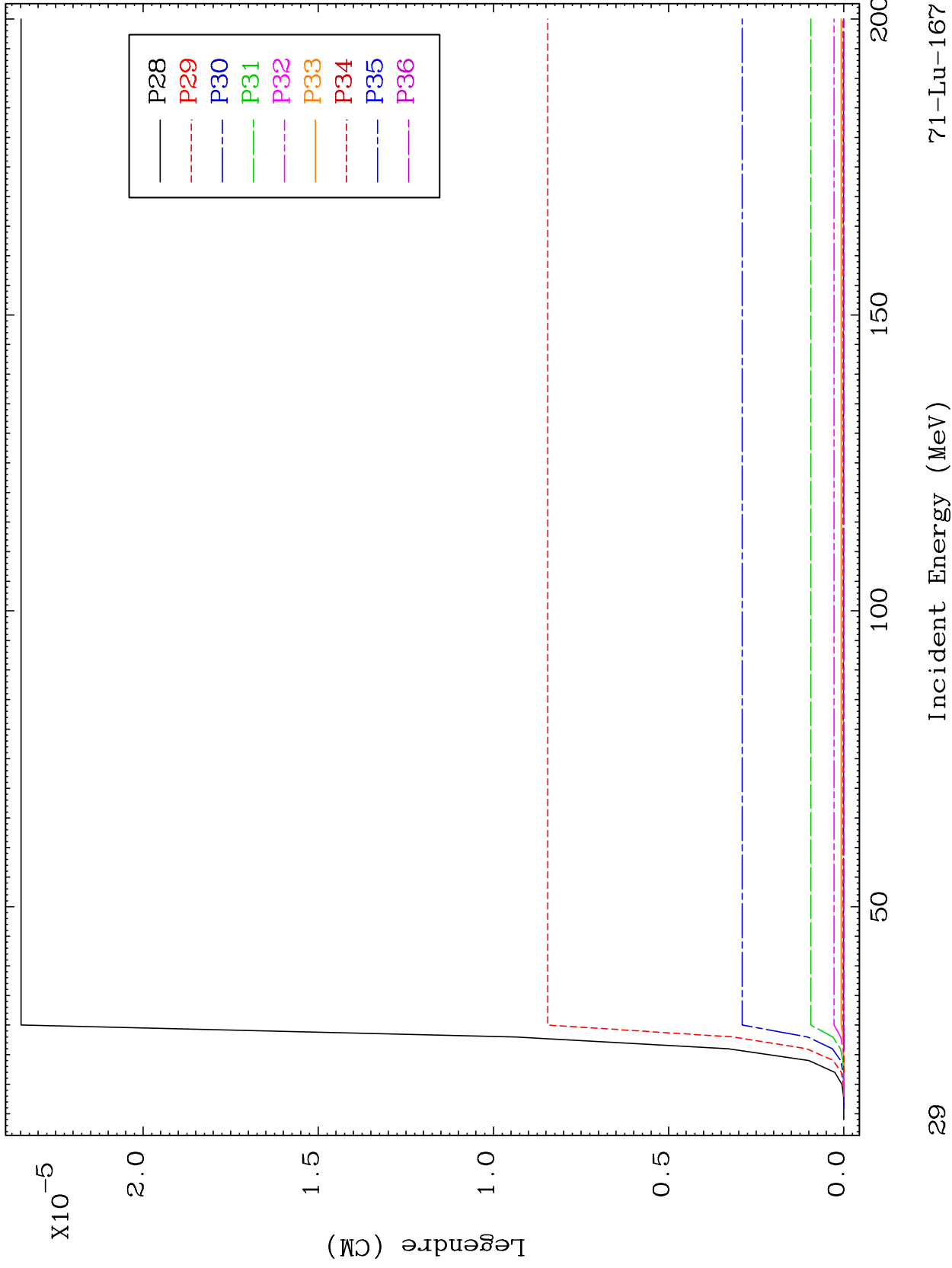


27

Incident Energy (MeV)

71-Lu-167

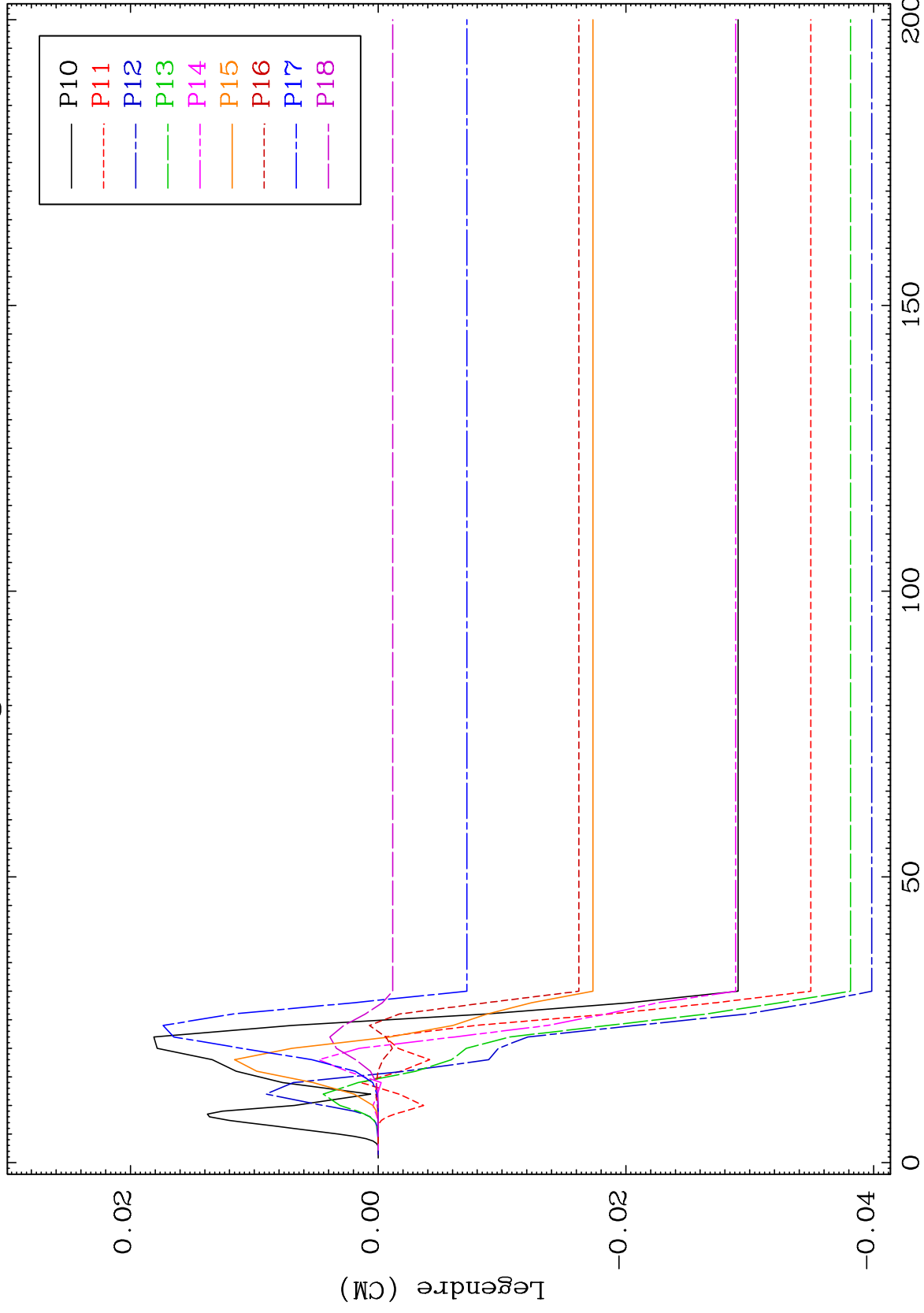




MAT 7101

MT= 54 (n,n') Level
Legendre Coefficients

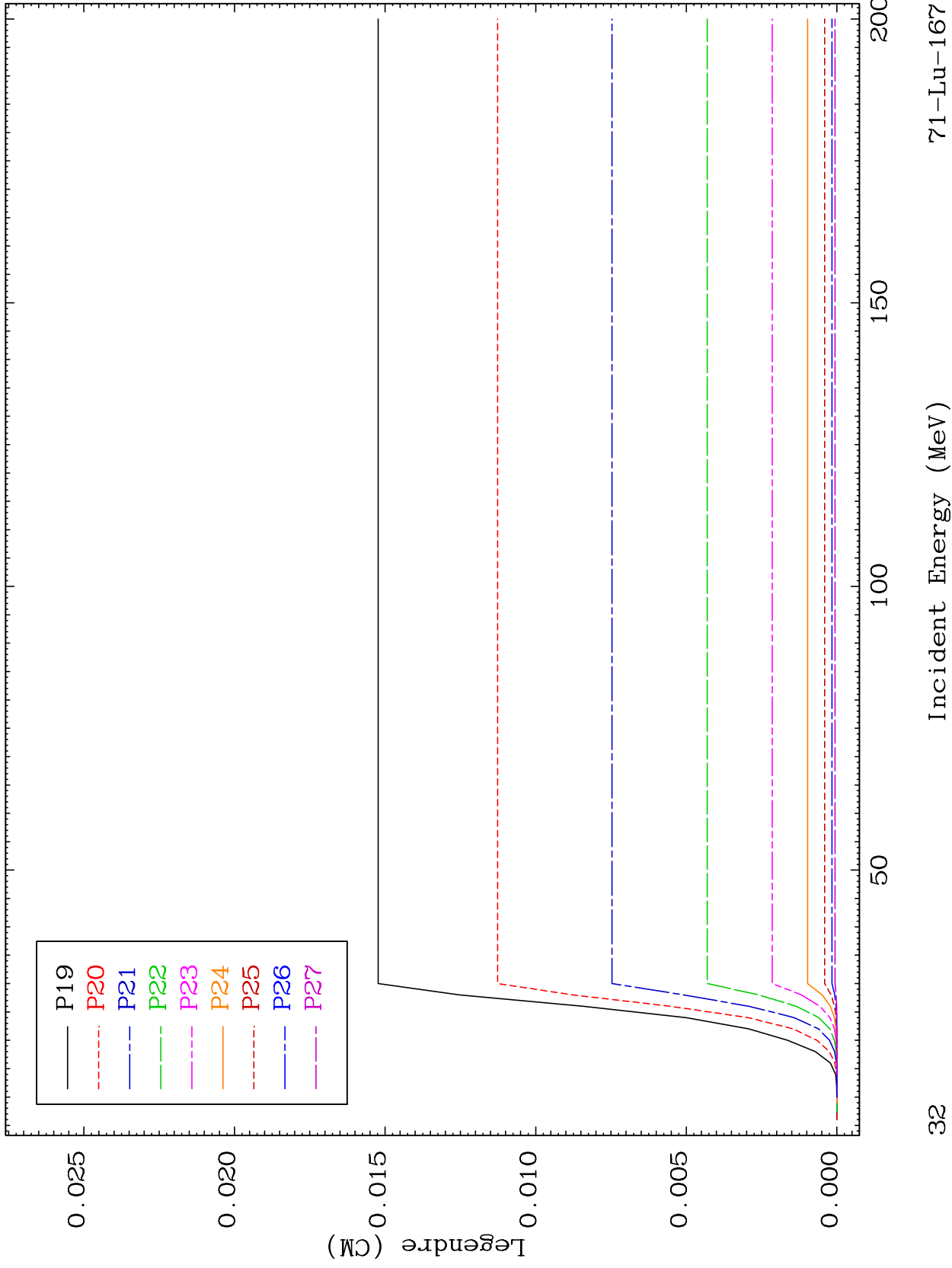
71-Lu-167



31

Incident Energy (MeV)

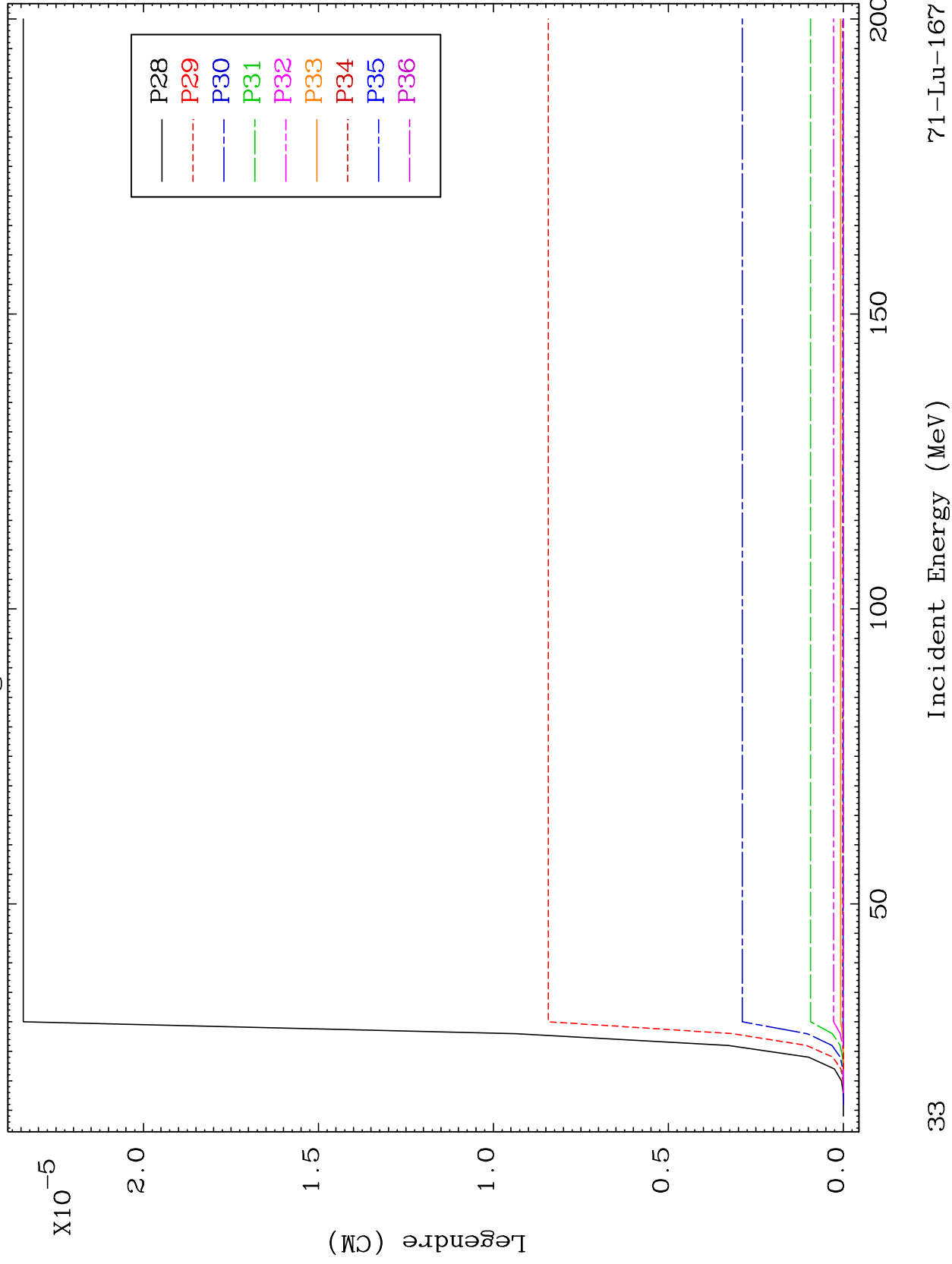
71-Lu-167



MAT 7101

MT= 54 (n,n') Level
Legendre Coefficients

71-Lu-167



33

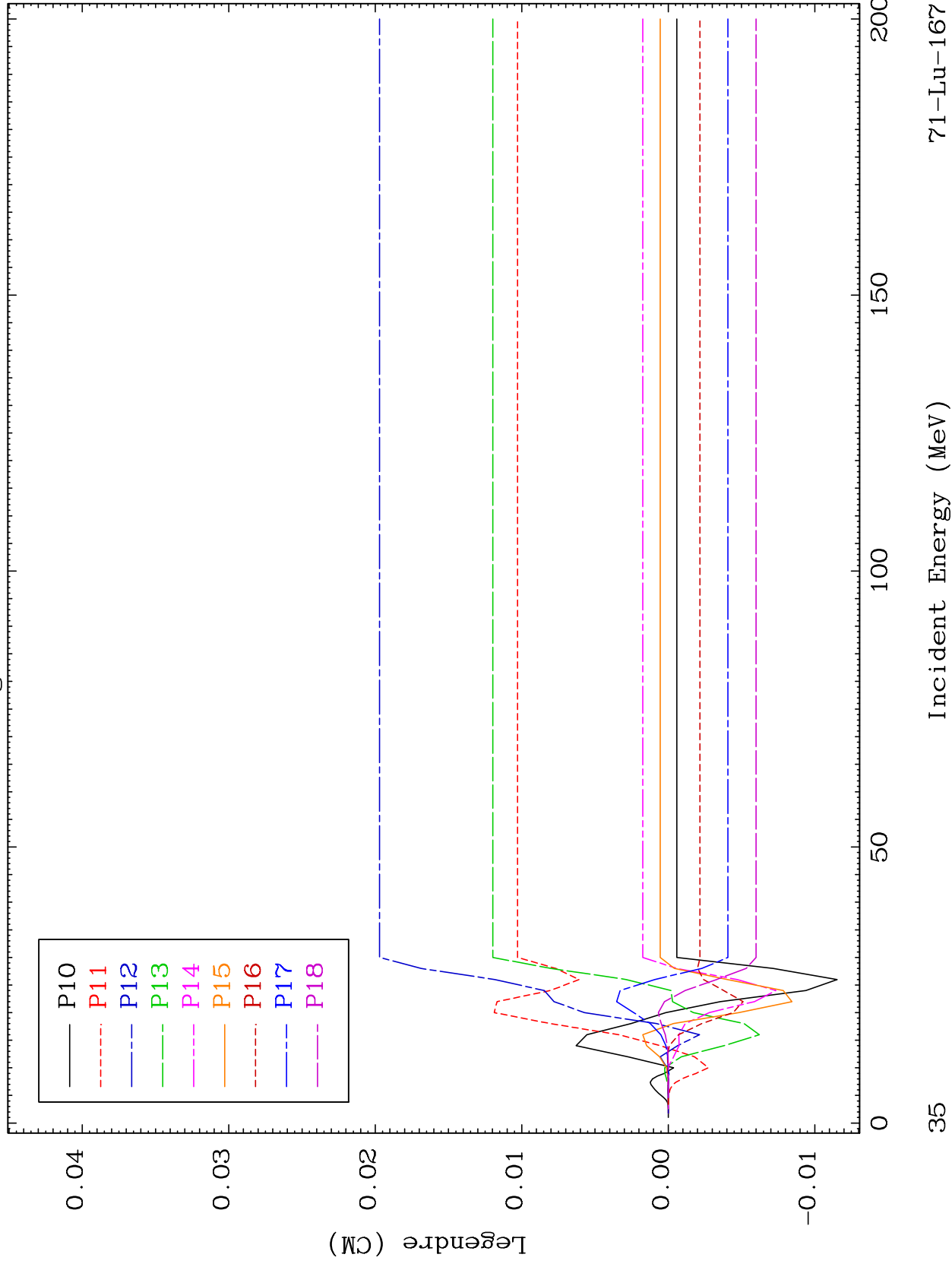
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 55 (n,n') Level
Legendre Coefficients

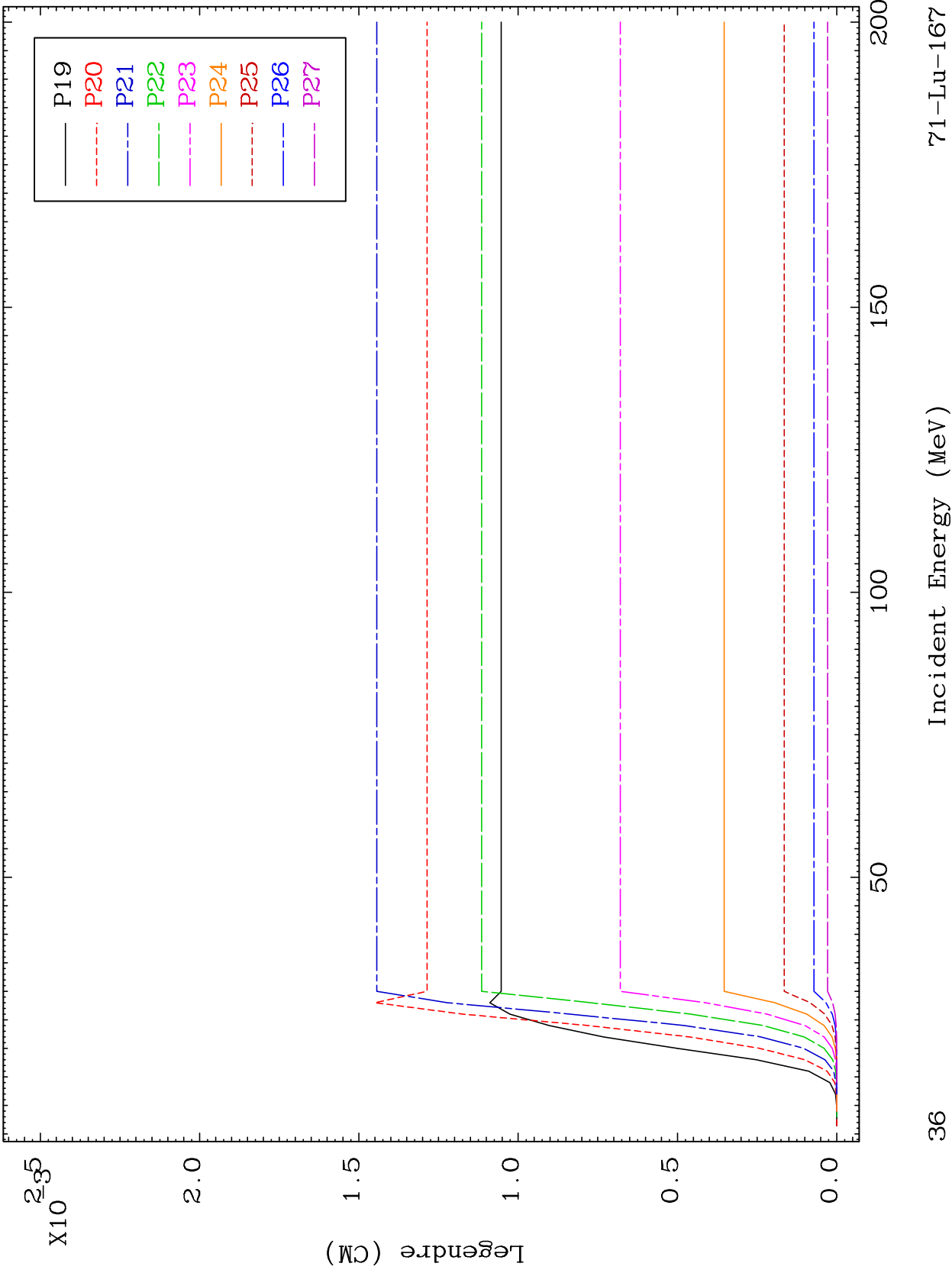
71-Lu-167

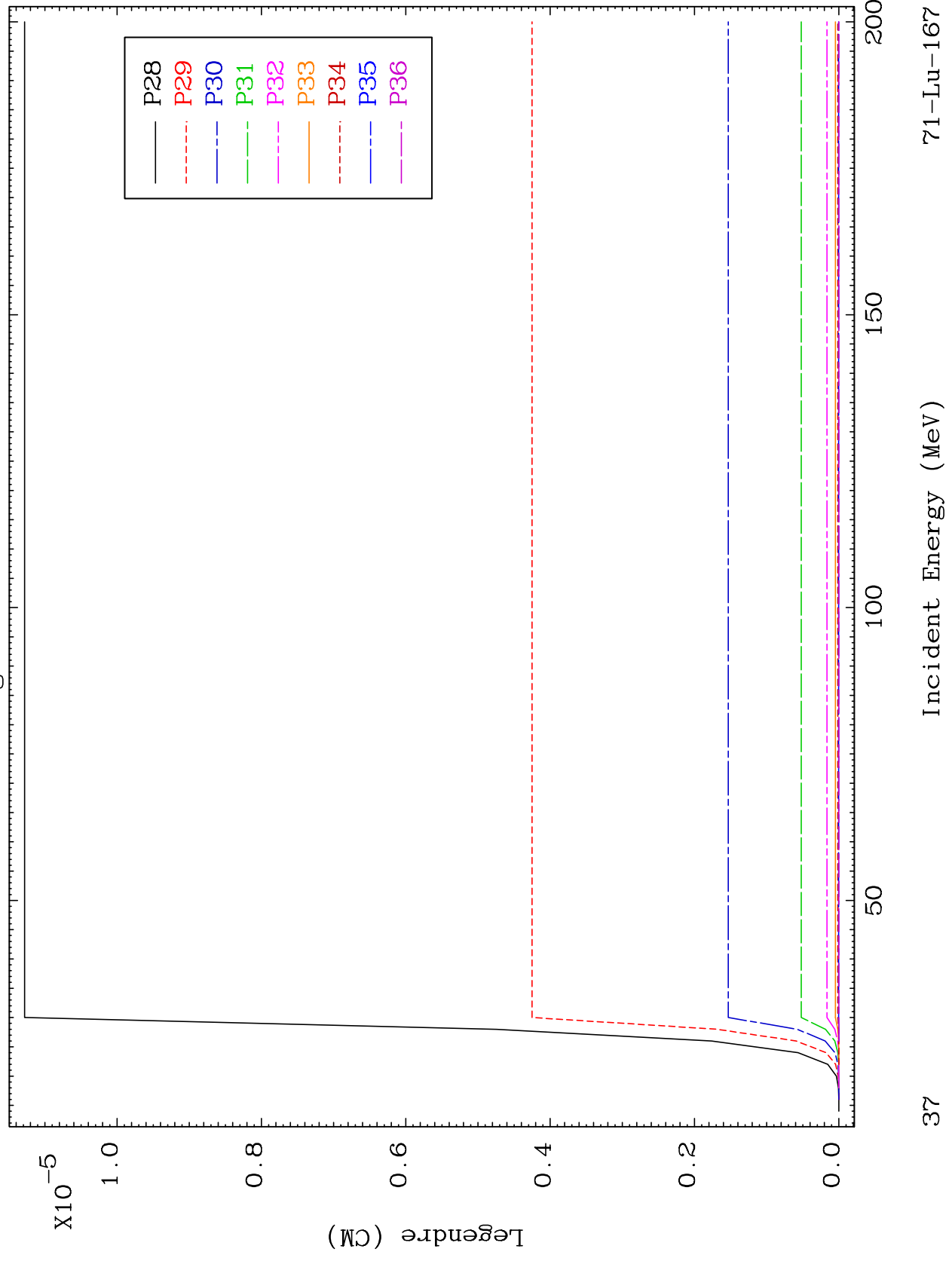


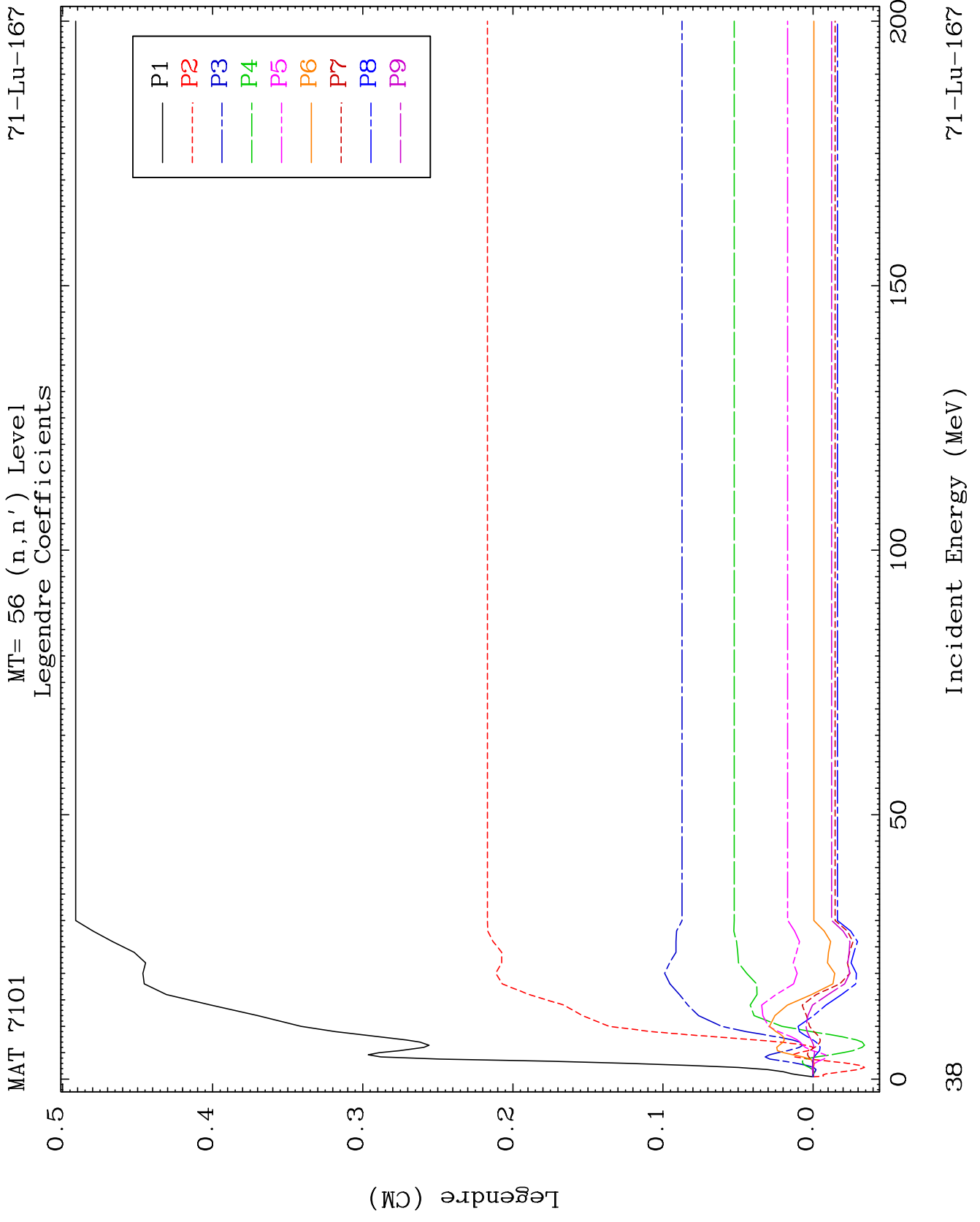
71-Lu-167

Incident Energy (MeV)

35



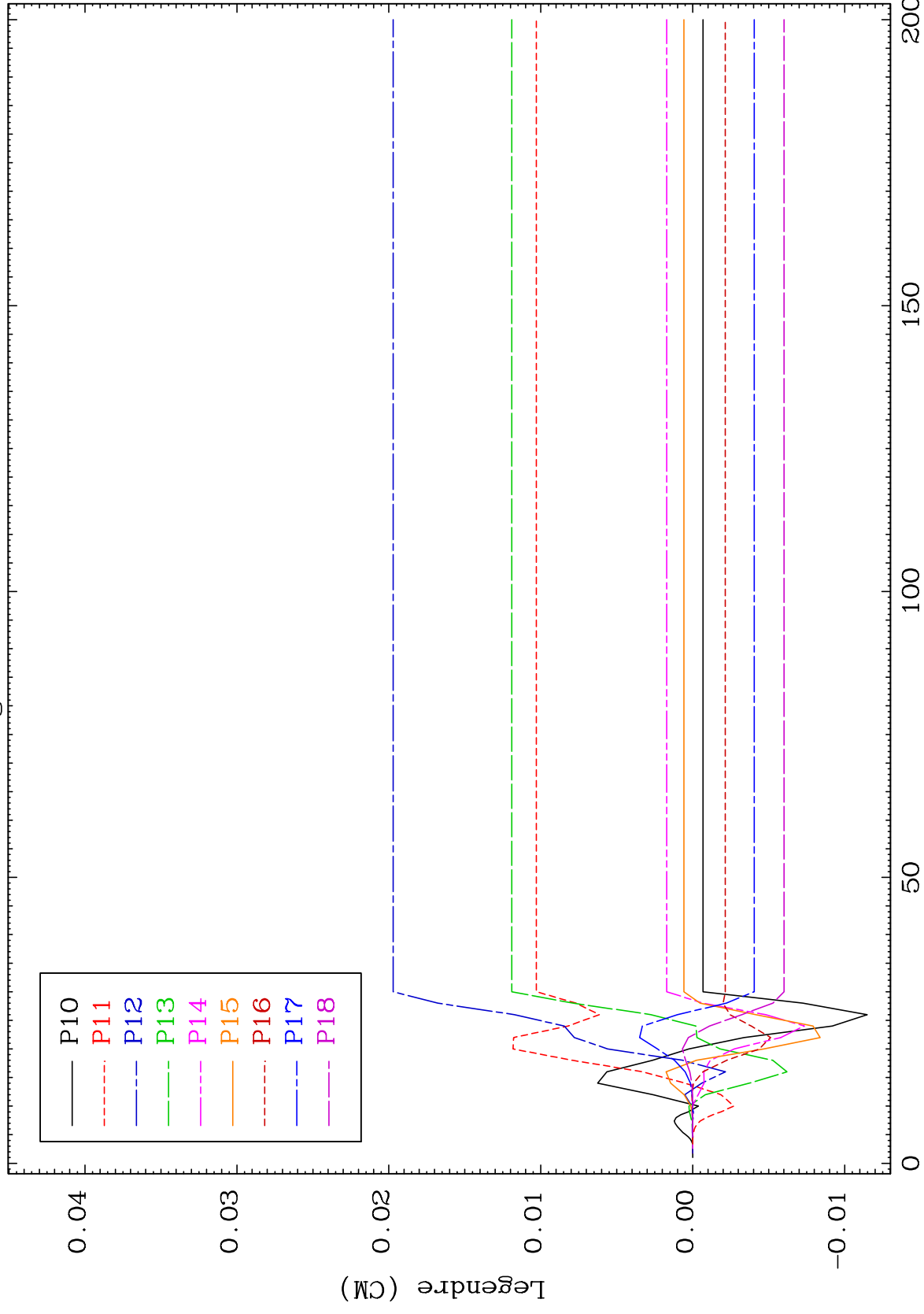




MAT 7101

MT= 56 (n,n') Level
Legendre Coefficients

71-Lu-167



39

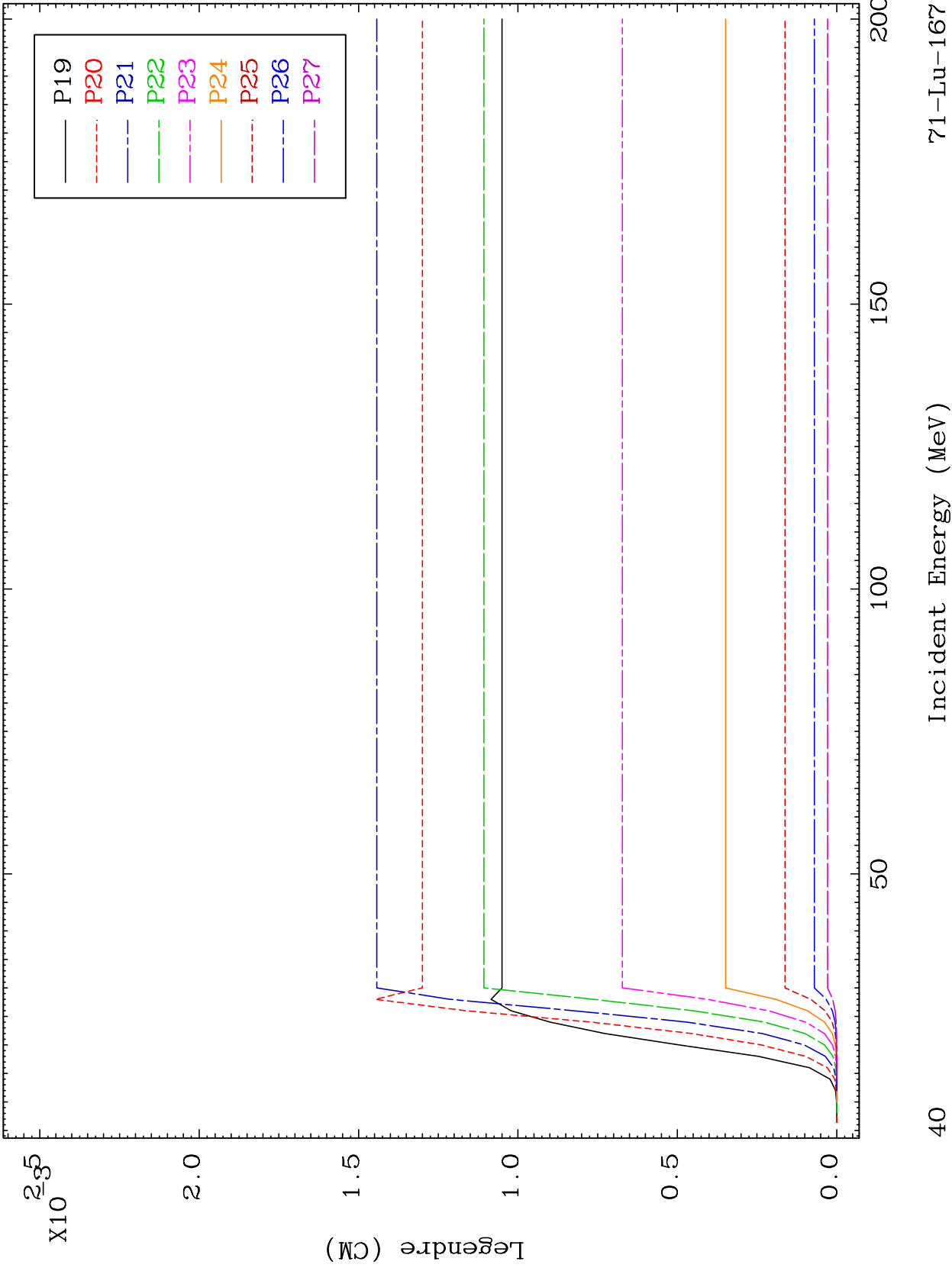
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 56 (n,n') Level
Legendre Coefficients

71-Lu-167



40

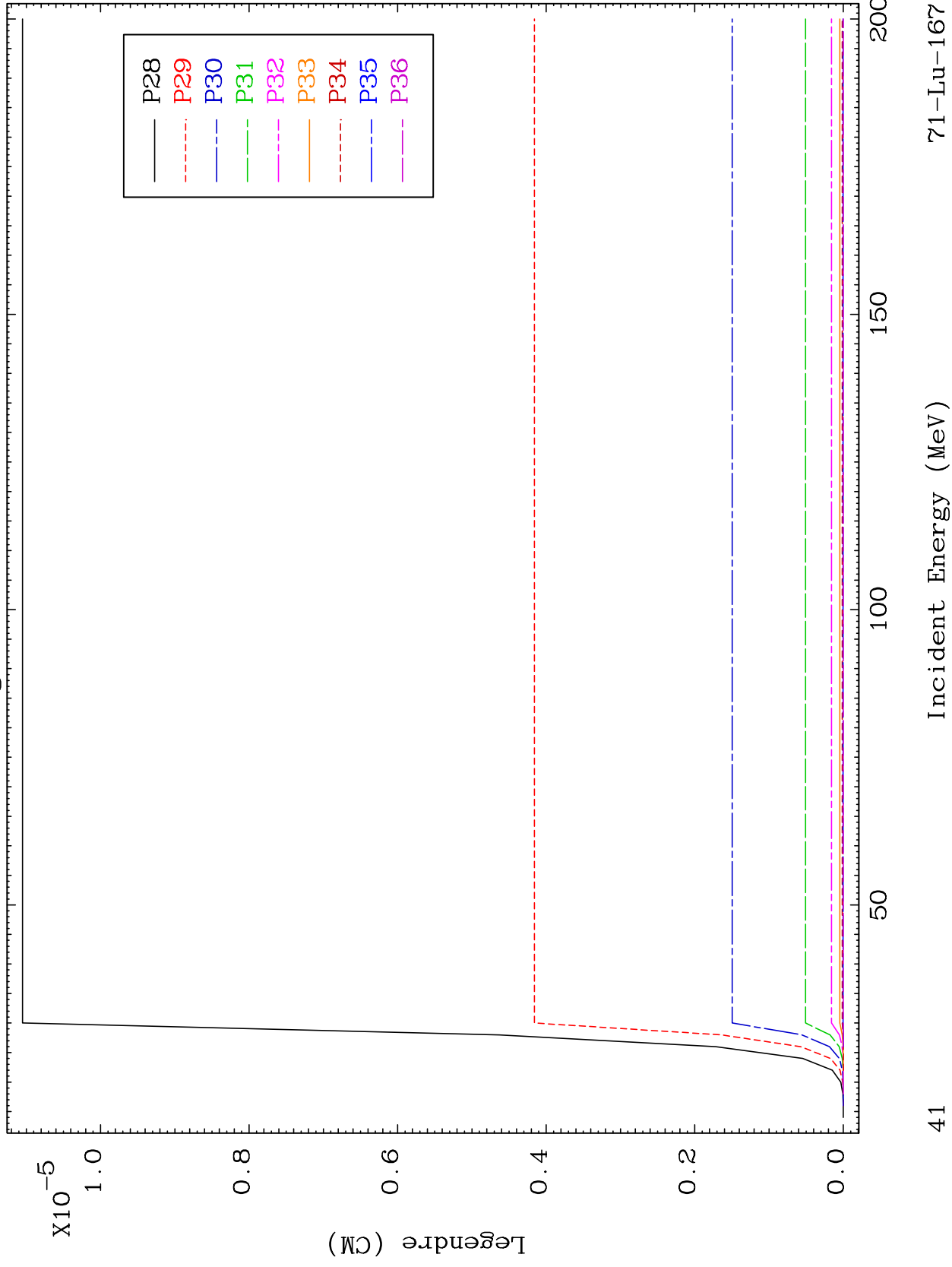
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 56 (n,n') Level
Legendre Coefficients

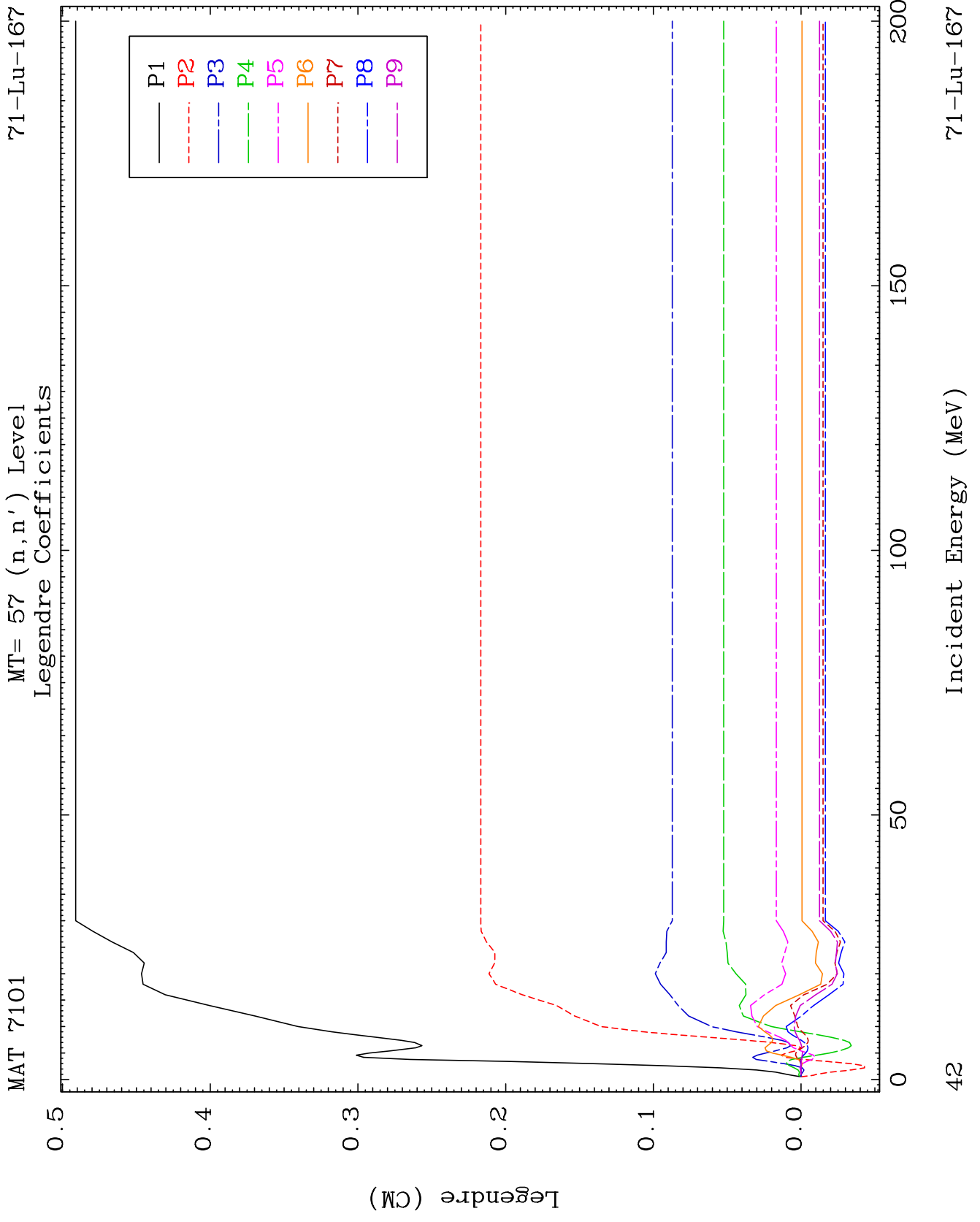
71-Lu-167



41

Incident Energy (MeV)

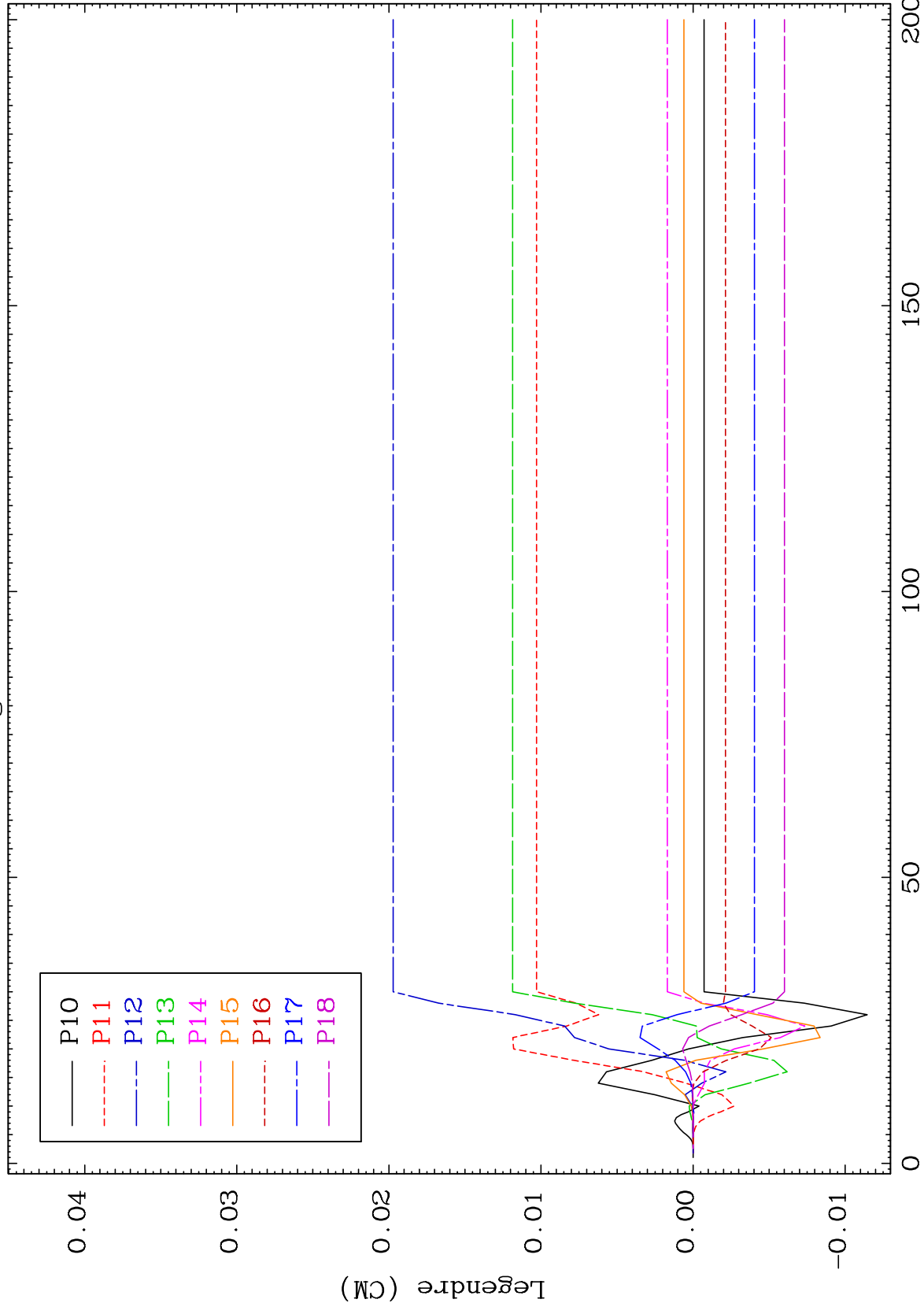
71-Lu-167



MAT 7101

MT= 57 (n,n') Level
Legendre Coefficients

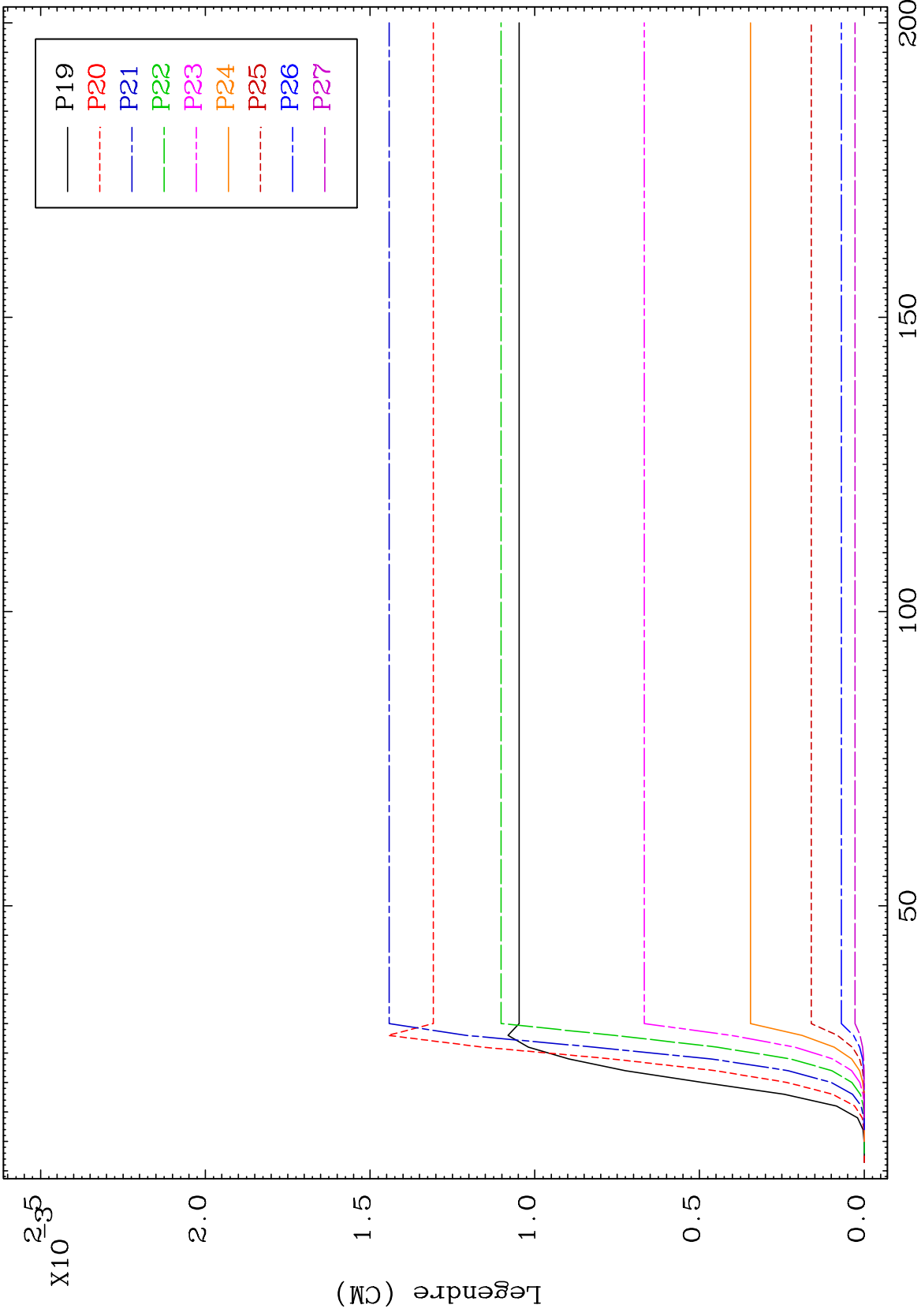
71-Lu-167



43

Incident Energy (MeV)

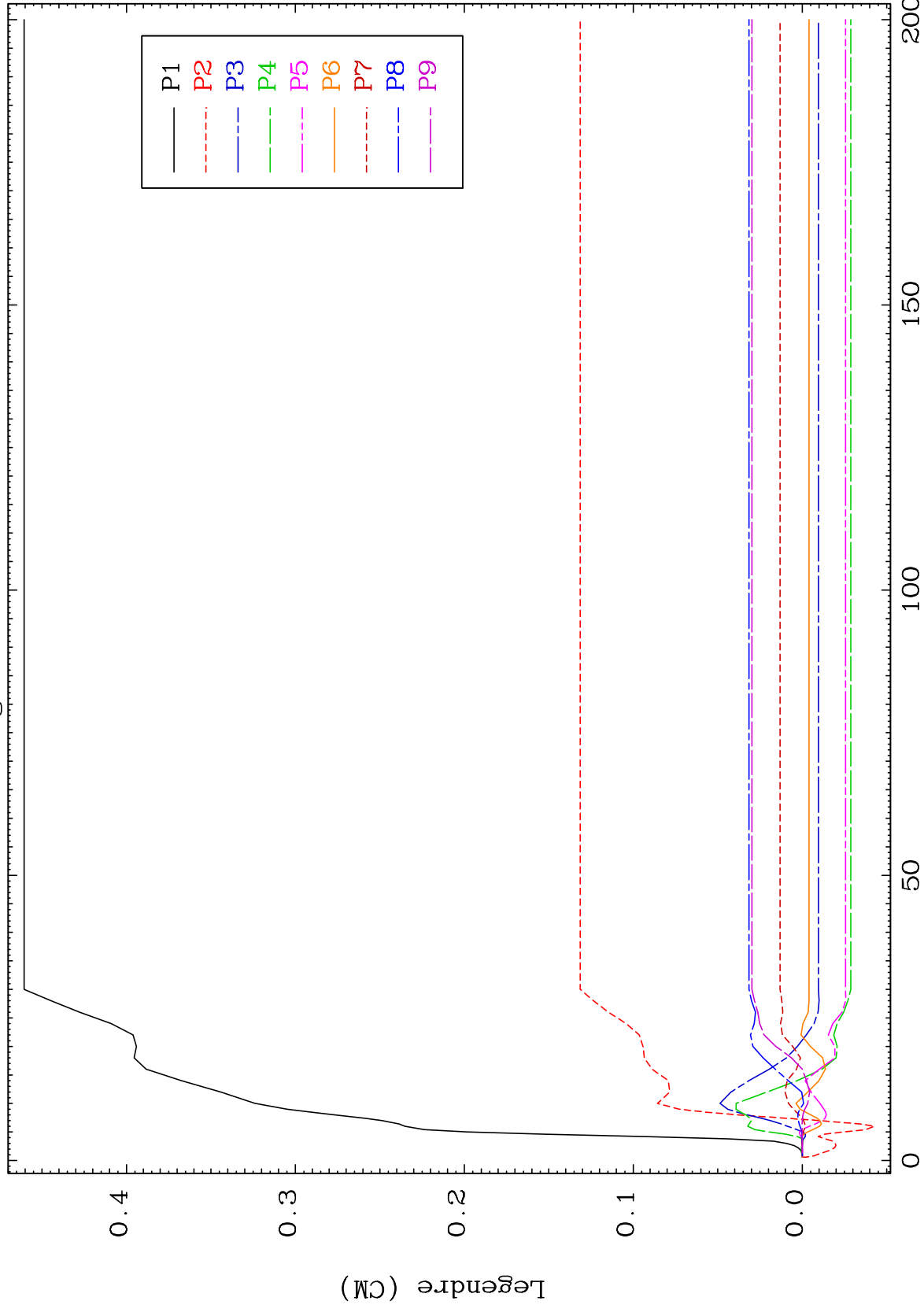
71-Lu-167



MAT 7101

MT= 58 (n,n') Level
Legendre Coefficients

71-Lu-167



46

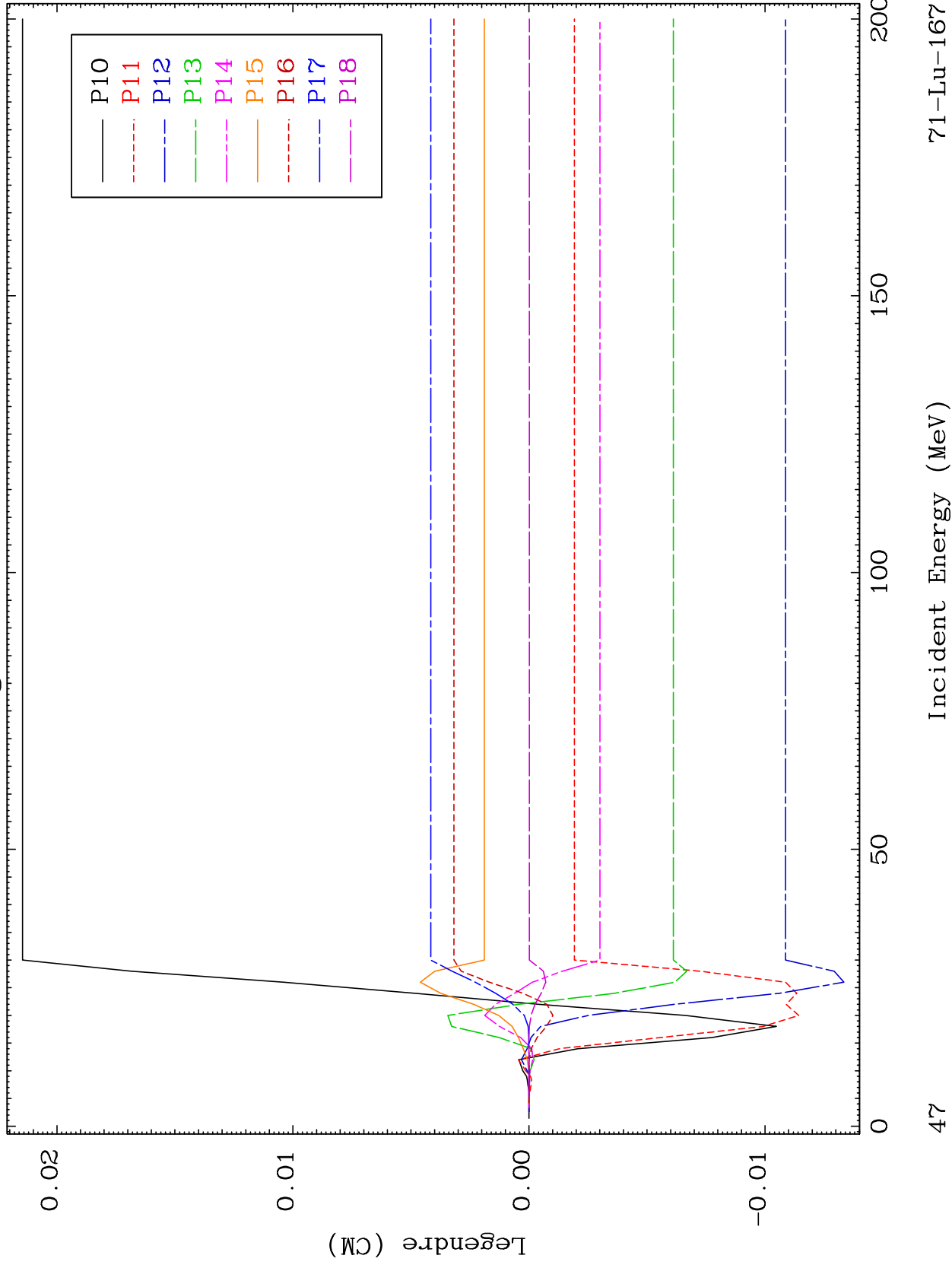
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 58 (n,n') Level
Legendre Coefficients

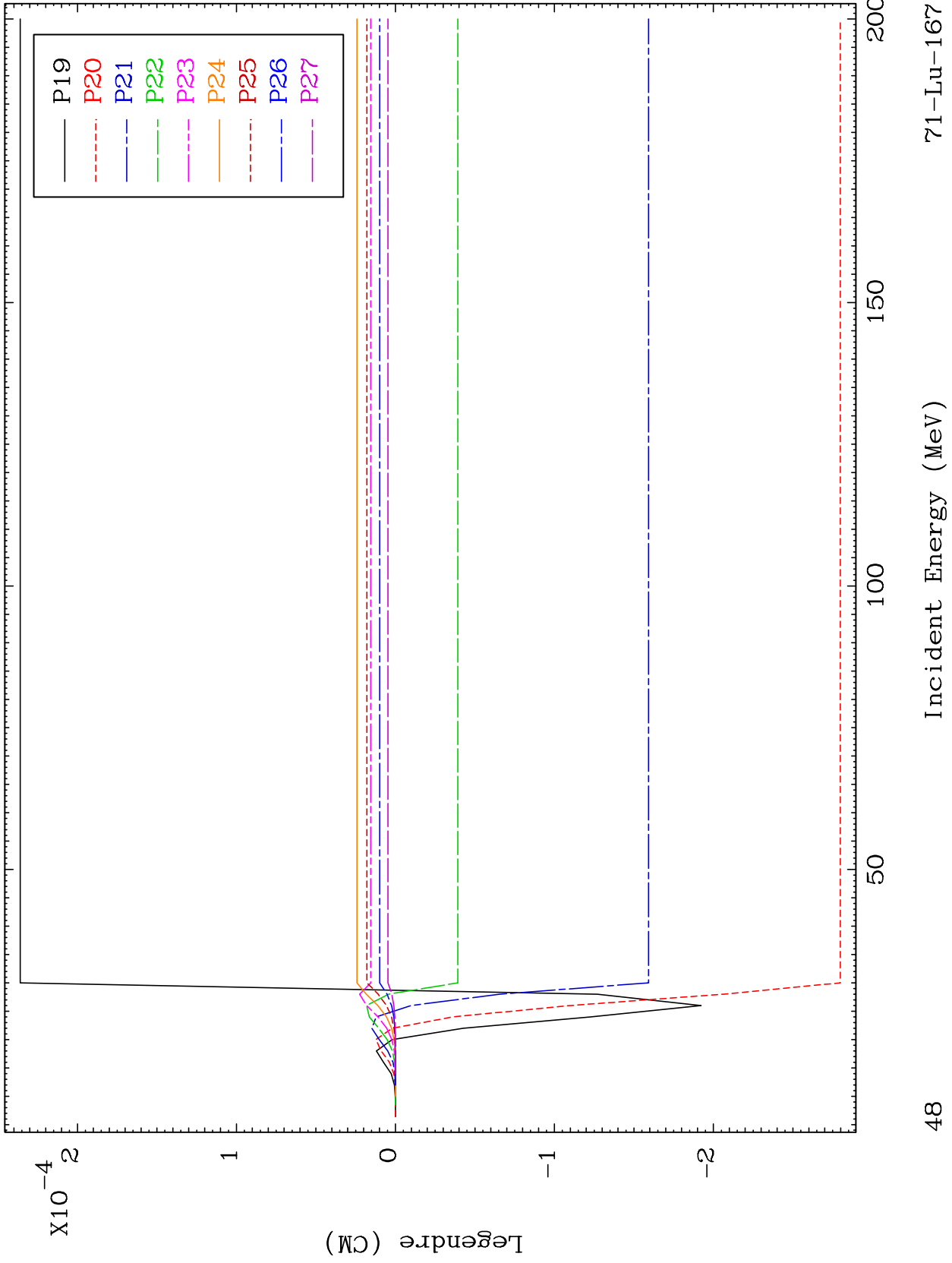
71-Lu-167



71-Lu-167

Incident Energy (MeV)

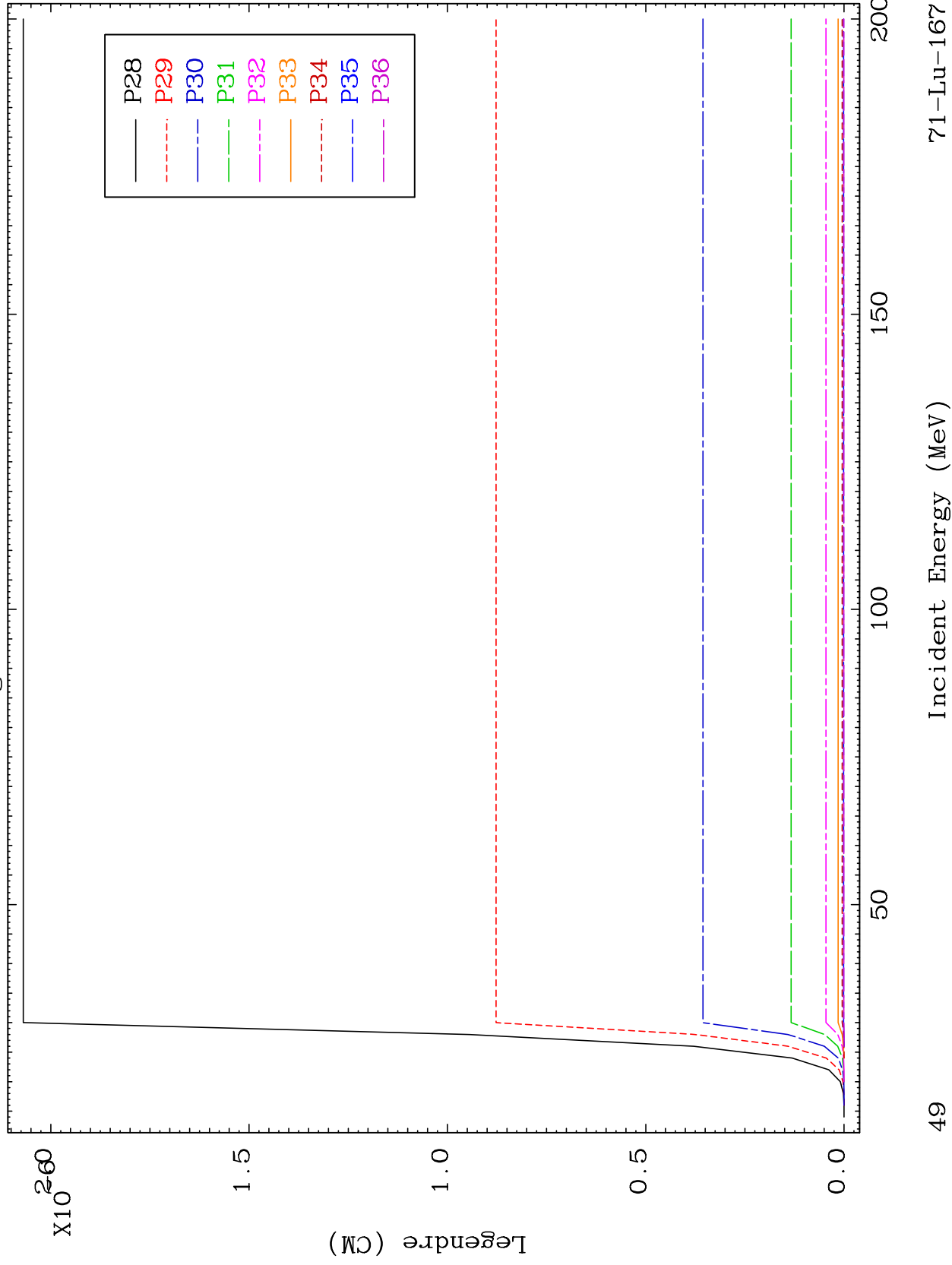
47



MAT 7101

MT= 58 (n,n') Level
Legendre Coefficients

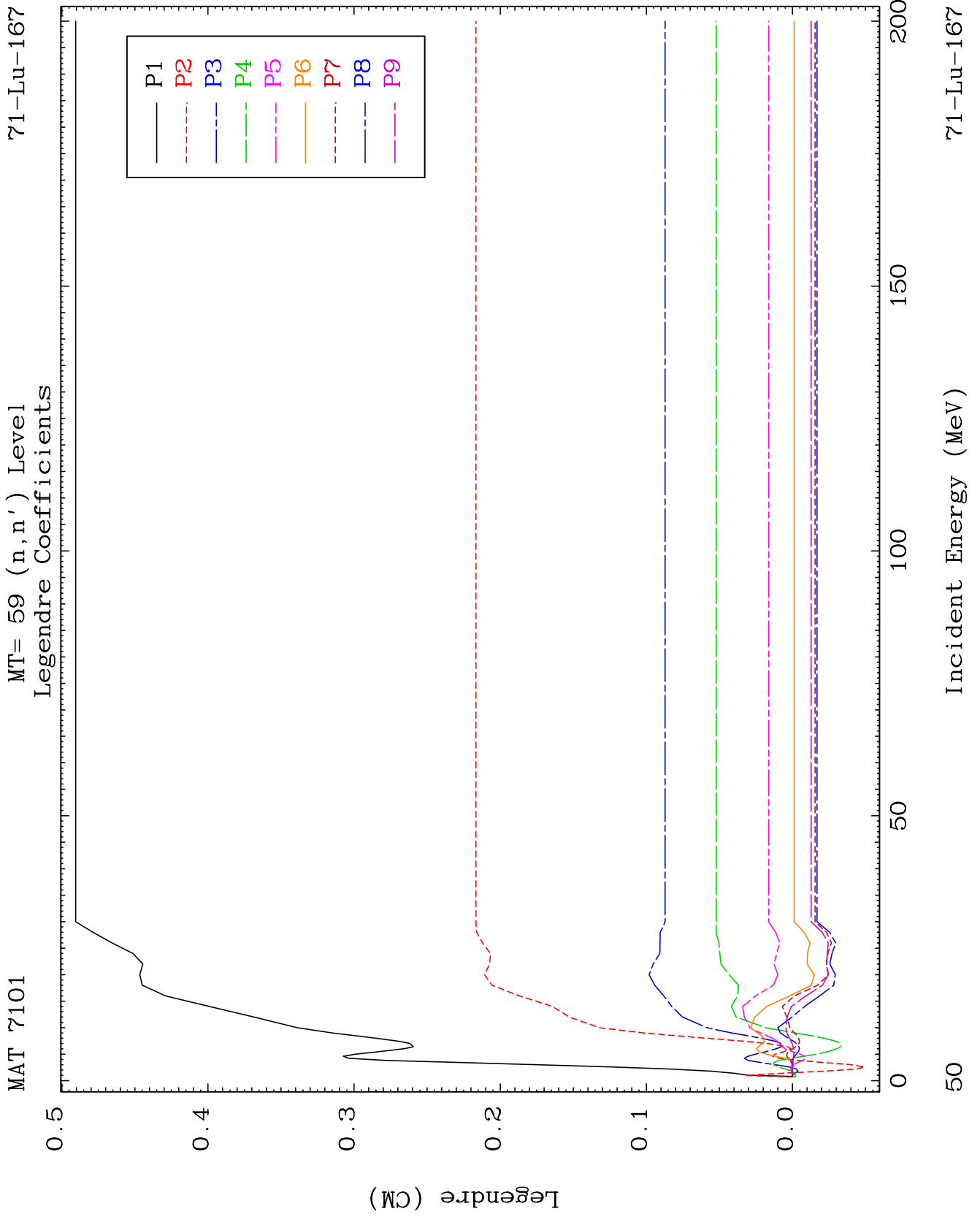
71-Lu-167



49

Incident Energy (MeV)

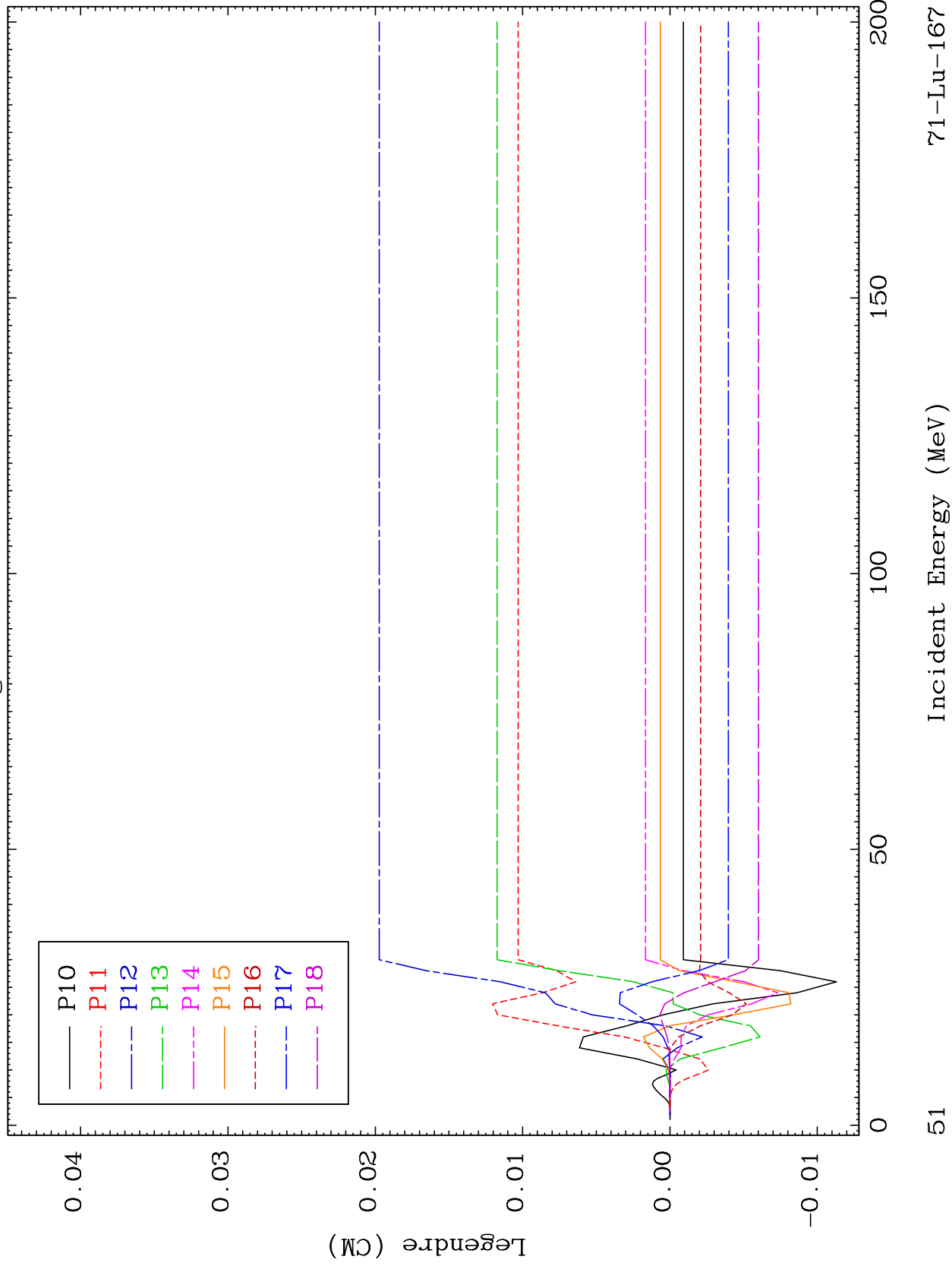
71-Lu-167



MAT 7101

MT= 59 (n,n') Level
Legendre Coefficients

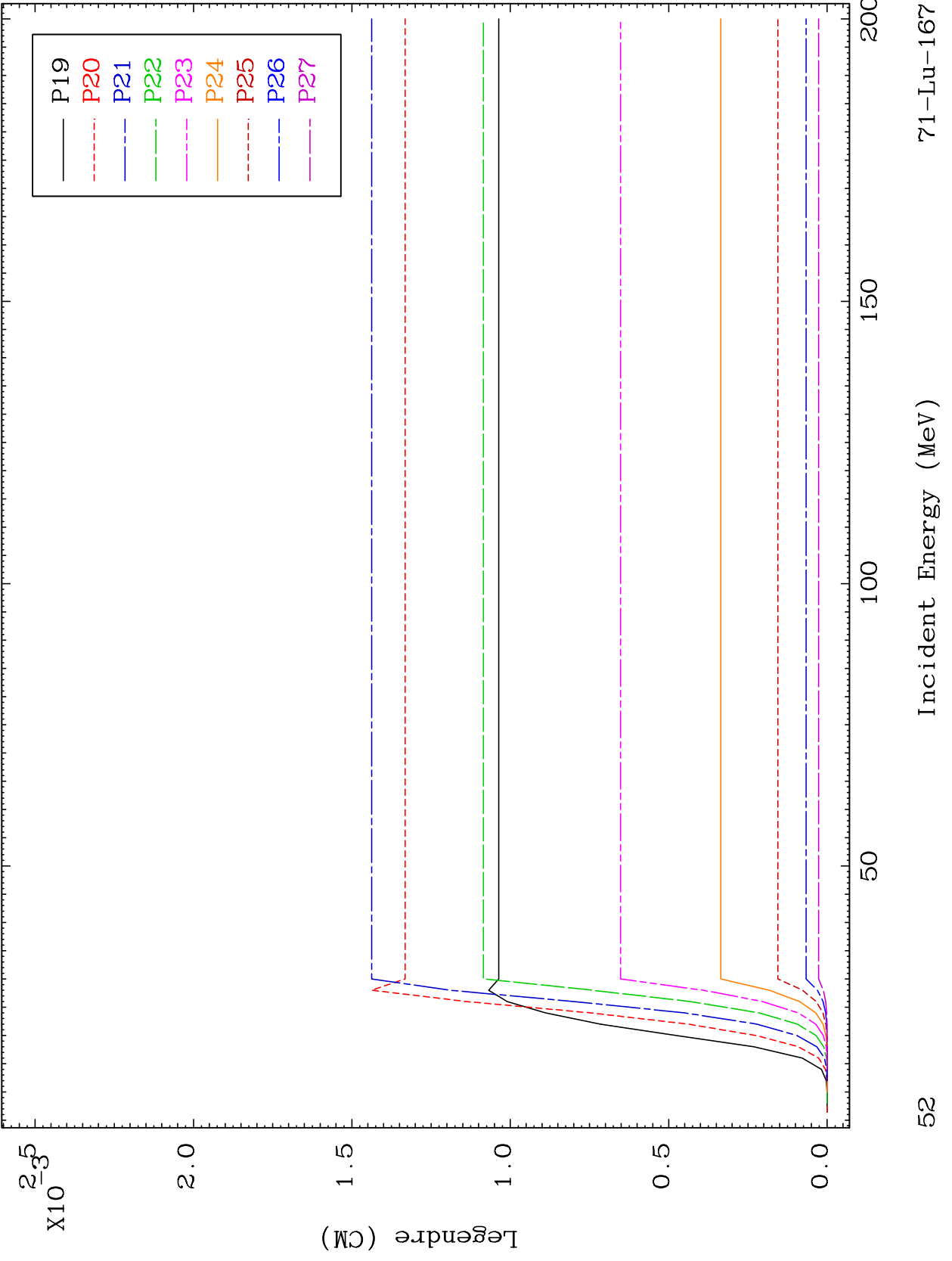
71-Lu-167

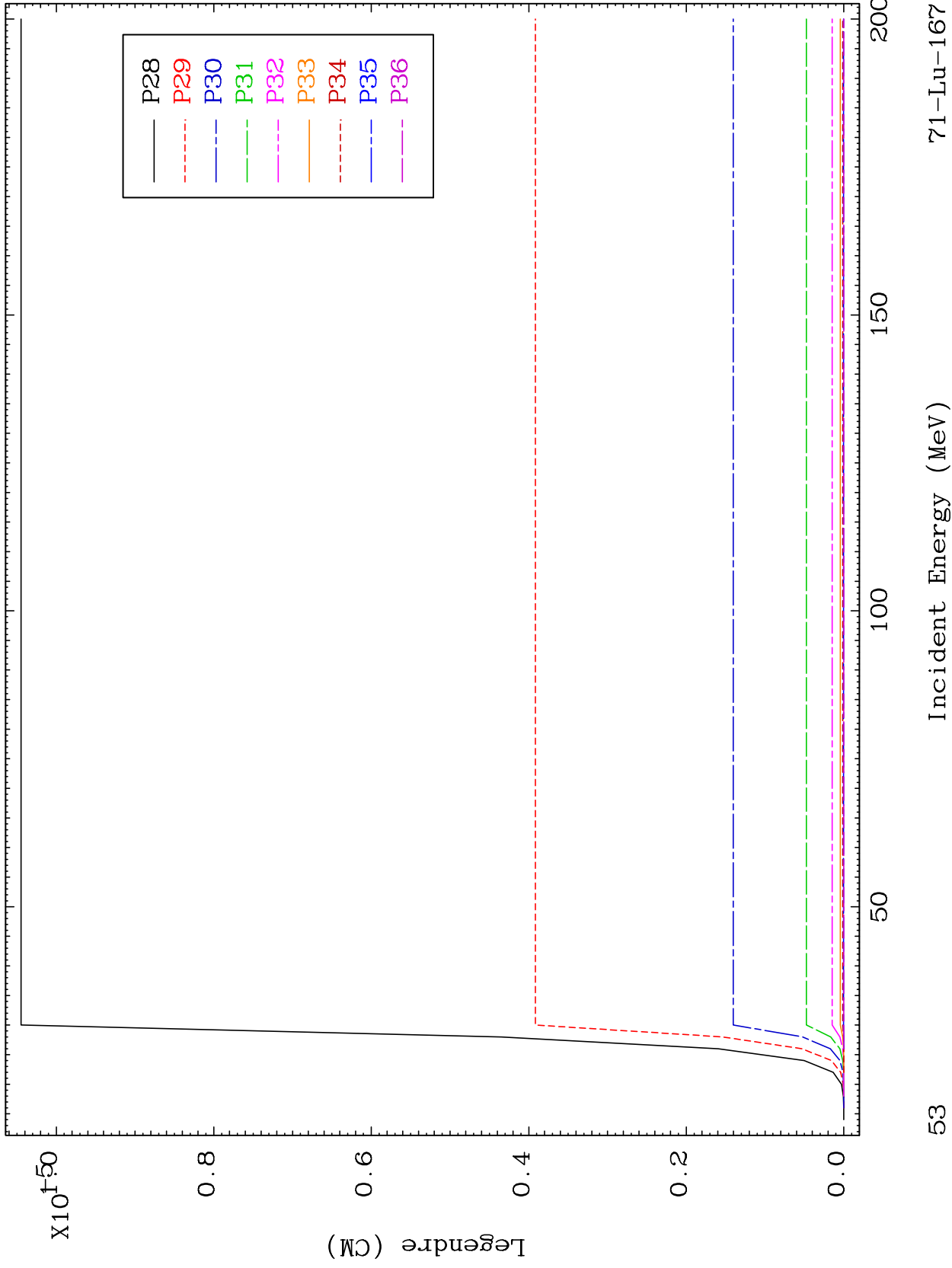


51

Incident Energy (MeV)

71-Lu-167

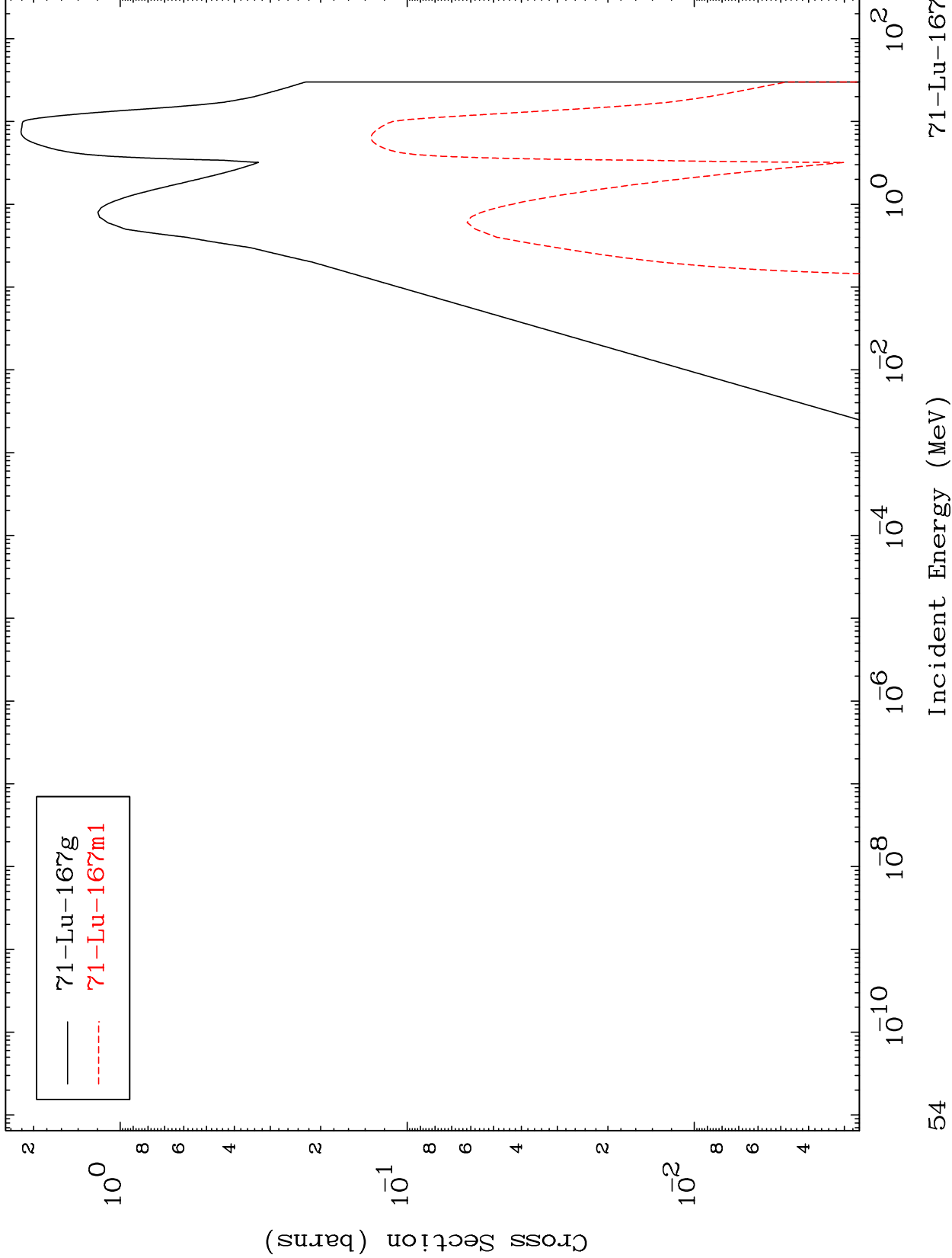




MAT 7101

Inelastic
Radionuclide Production Cross Section

⁷¹Lu-167



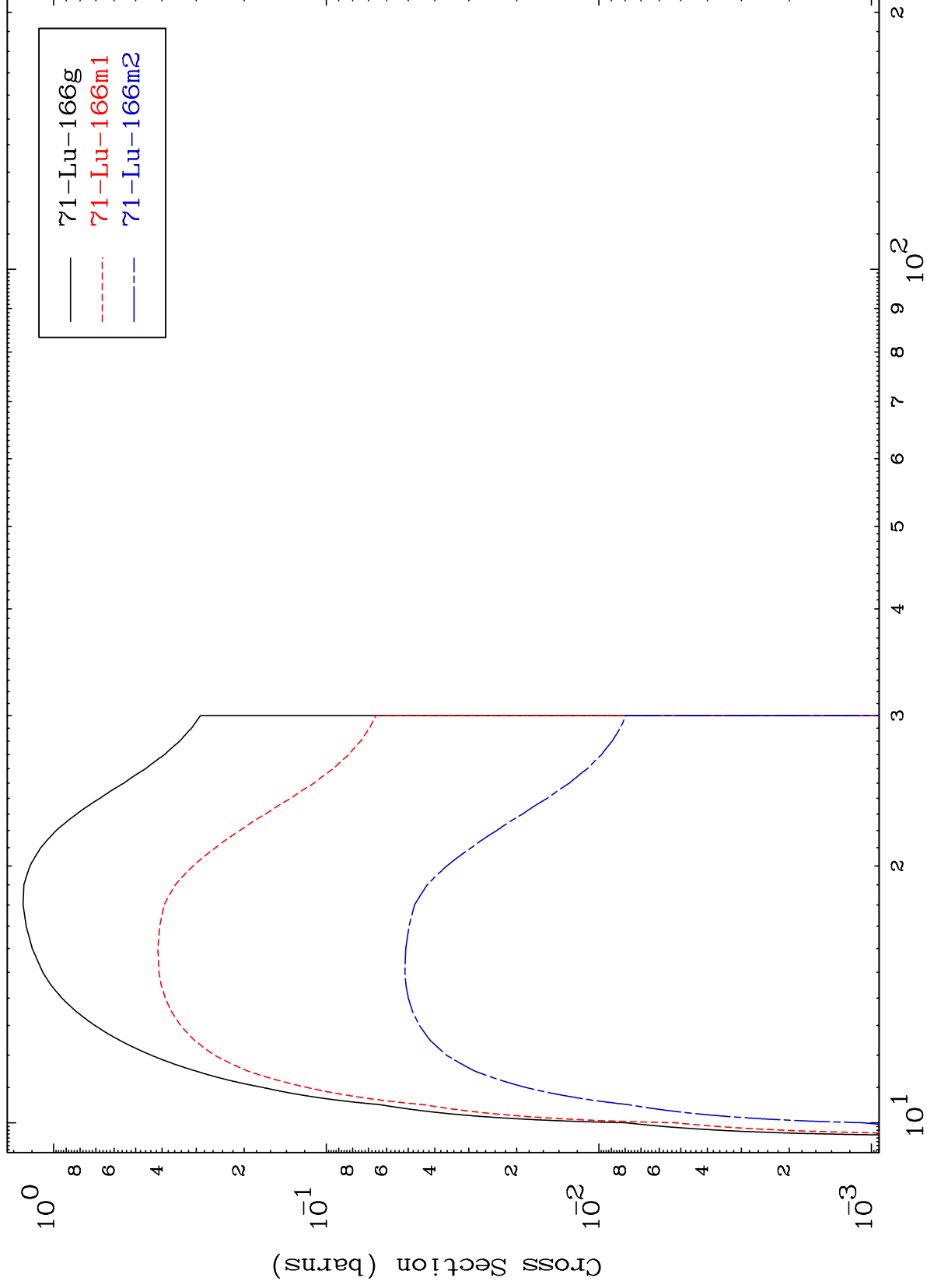
— ⁷¹Lu-167g
- - - ⁷¹Lu-167m1

MAT 7101

(n,2n)

71-Lu-167

Radionuclide Production Cross Section



Incident Energy (MeV)

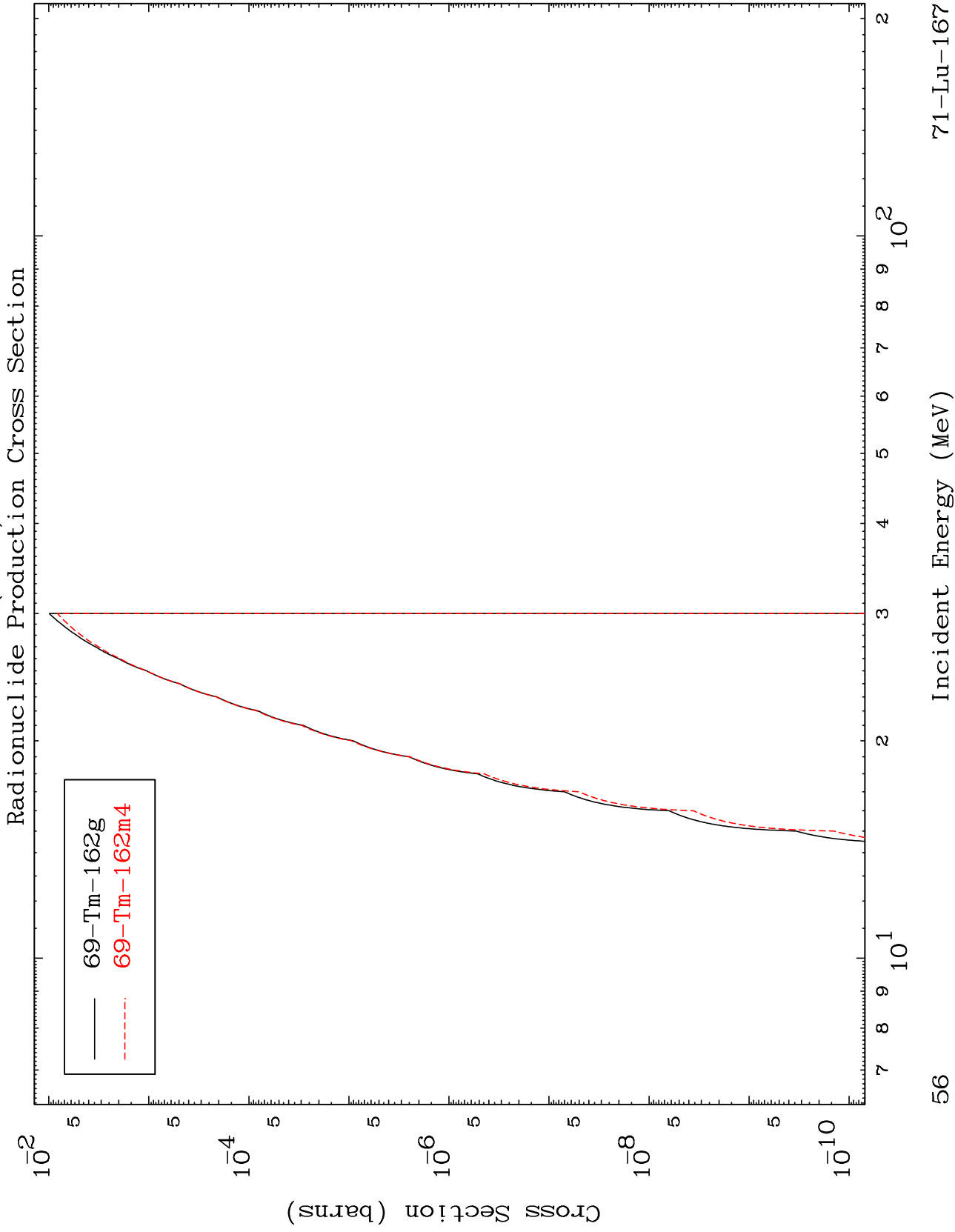
71-Lu-167

55

MAT 7101

$(n,2n) \alpha$

$^{71}\text{Lu-167}$



56

Incident Energy (MeV)

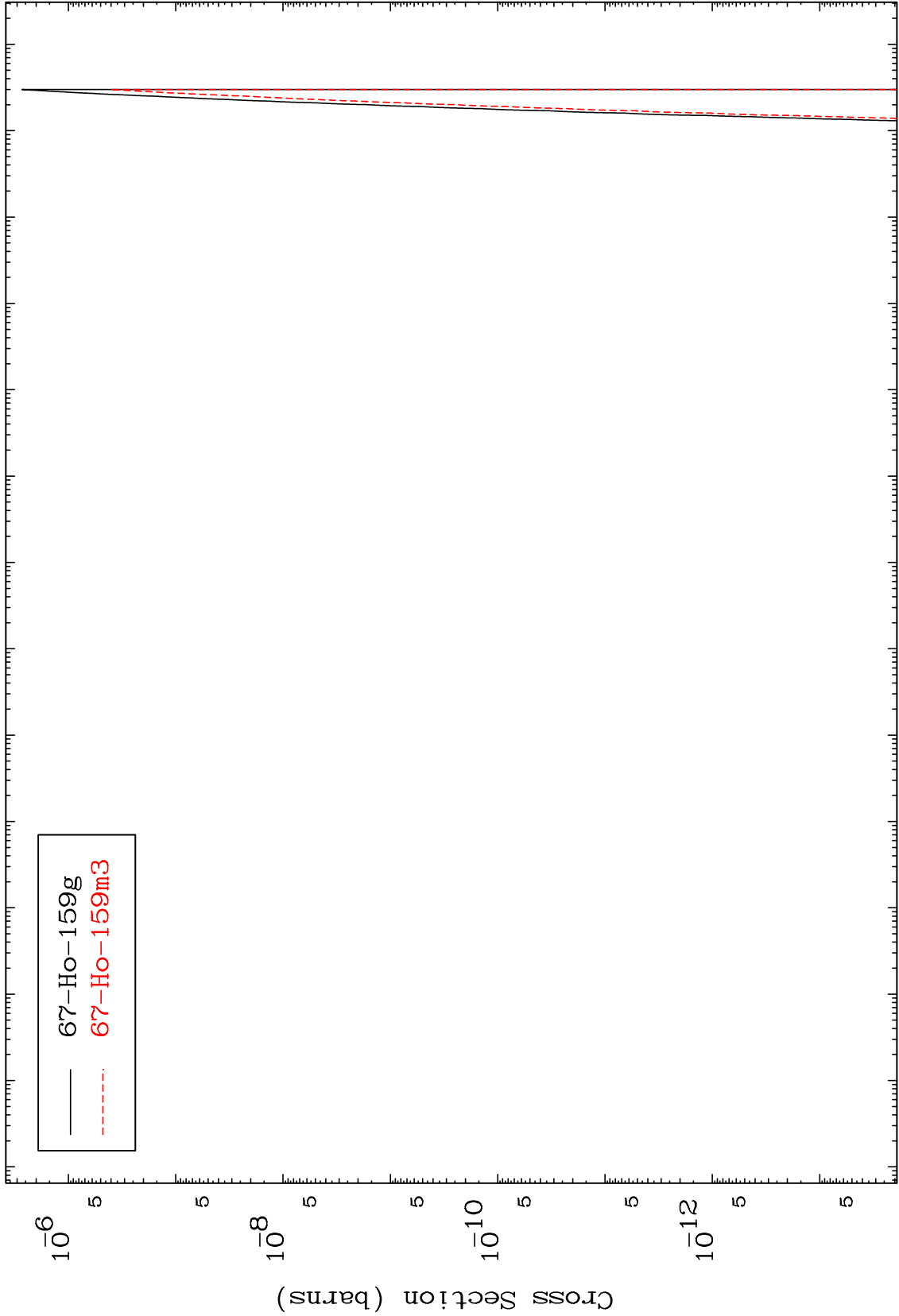
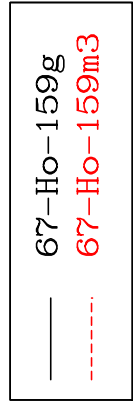
$^{71}\text{Lu-167}$

MAT 7101

(n,n') 2α

71-Lu-167

Radionuclide Production Cross Section



71-Lu-167

10⁰

10⁻²

10⁻⁴

10⁻⁶

10⁻⁸

10⁻¹⁰

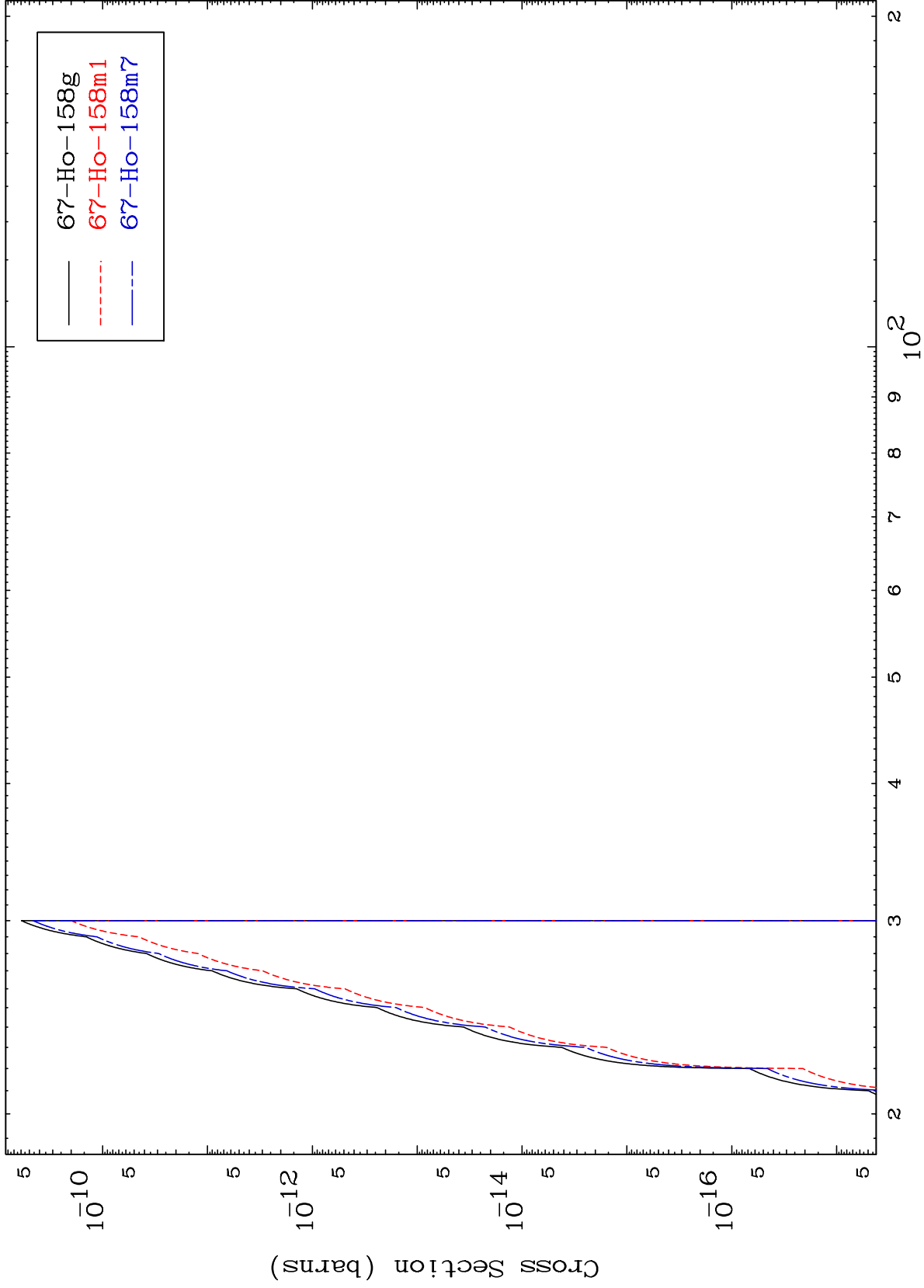
10⁻¹²

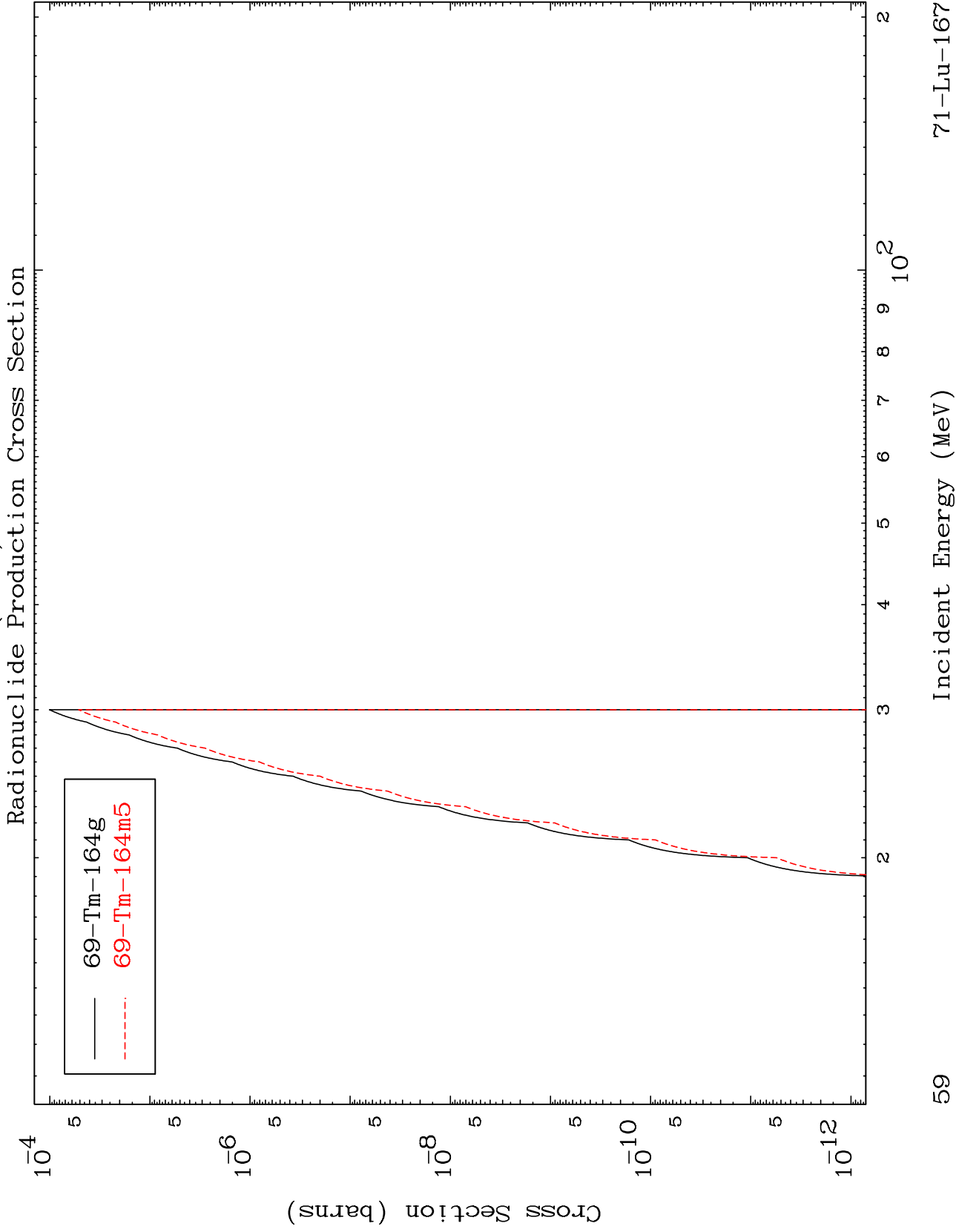
Incident Energy (MeV)

71-Lu-167

57

Radionuclide Production Cross Section

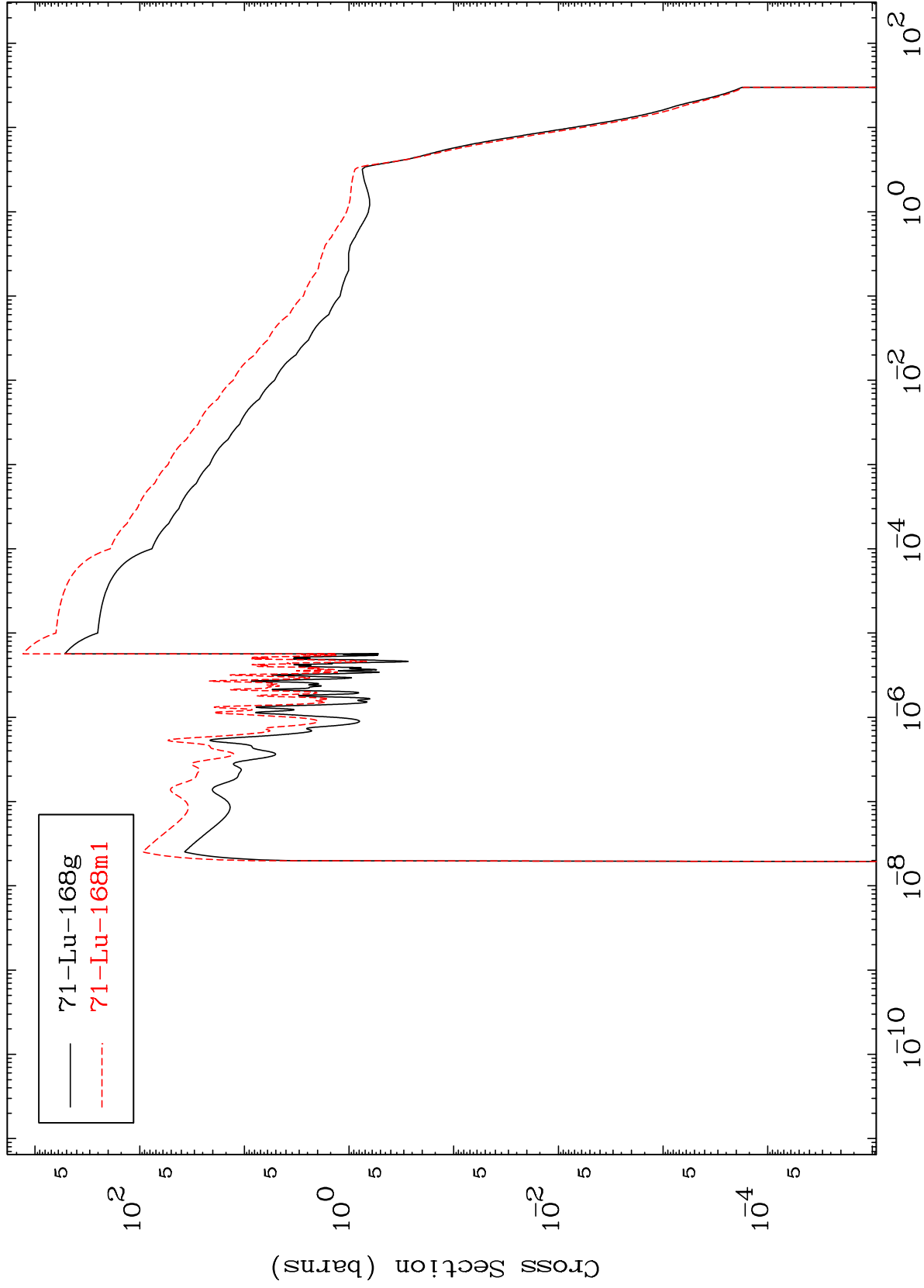




MAT 7101

71-Lu-167

Radionuclide Production Cross Section



60

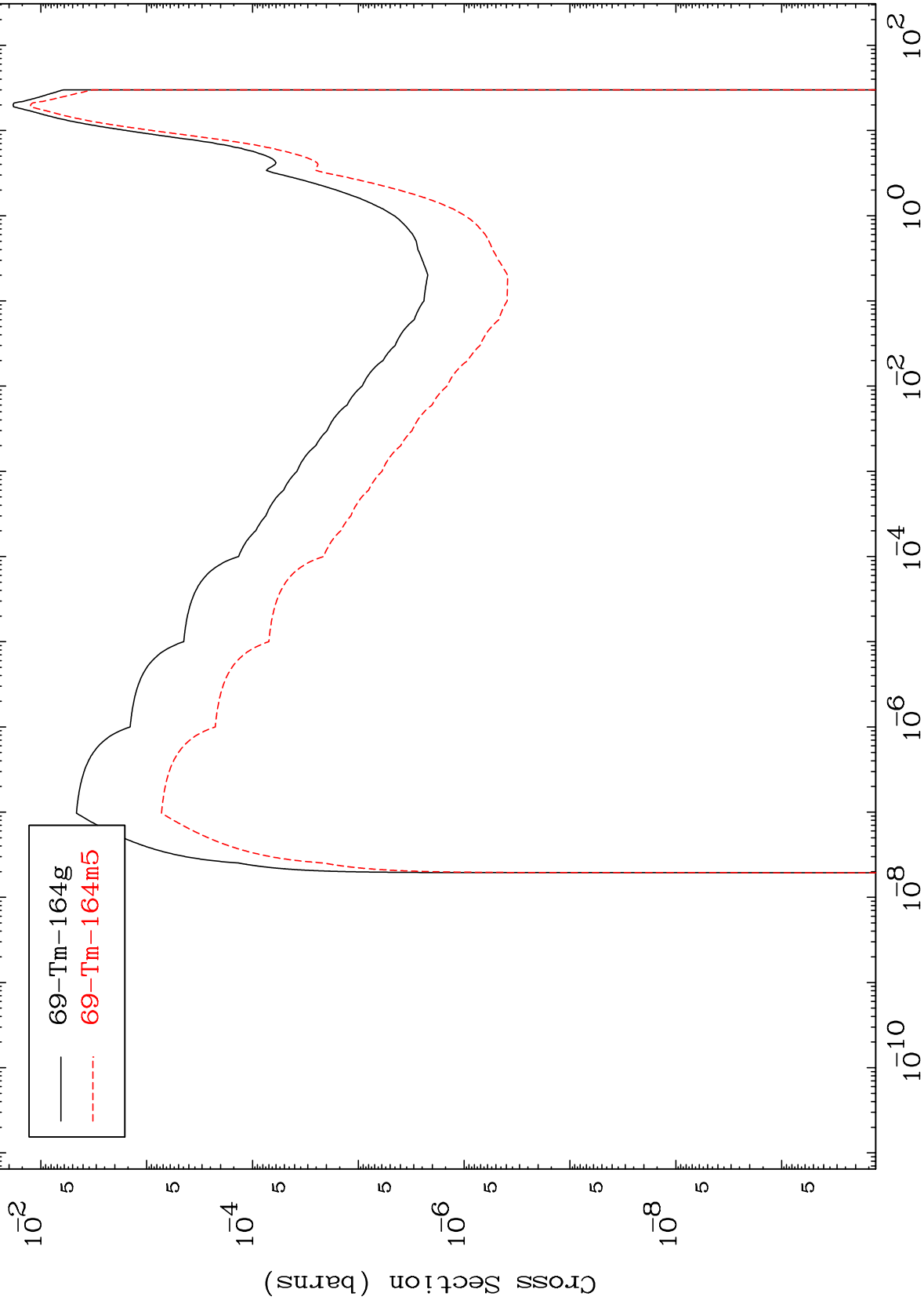
Incident Energy (MeV)

71-Lu-167

MAT 7101

71-Lu-167

Radionuclide Production Cross Section
(n, α)



61

Incident Energy (MeV)

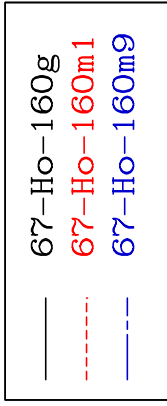
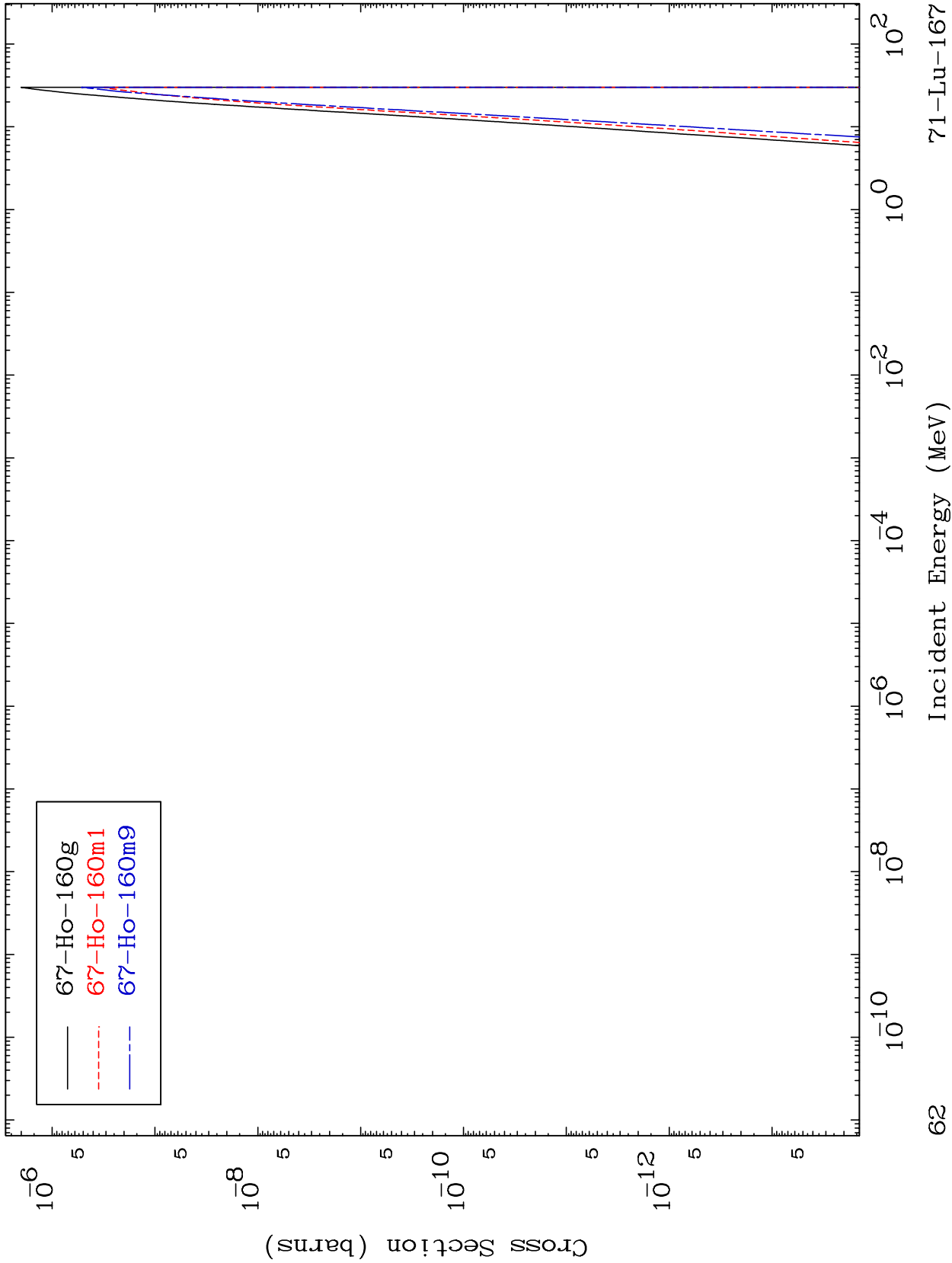
71-Lu-167

MAT 7101

(n,2α)

⁷¹Lu-167

Radionuclide Production Cross Section

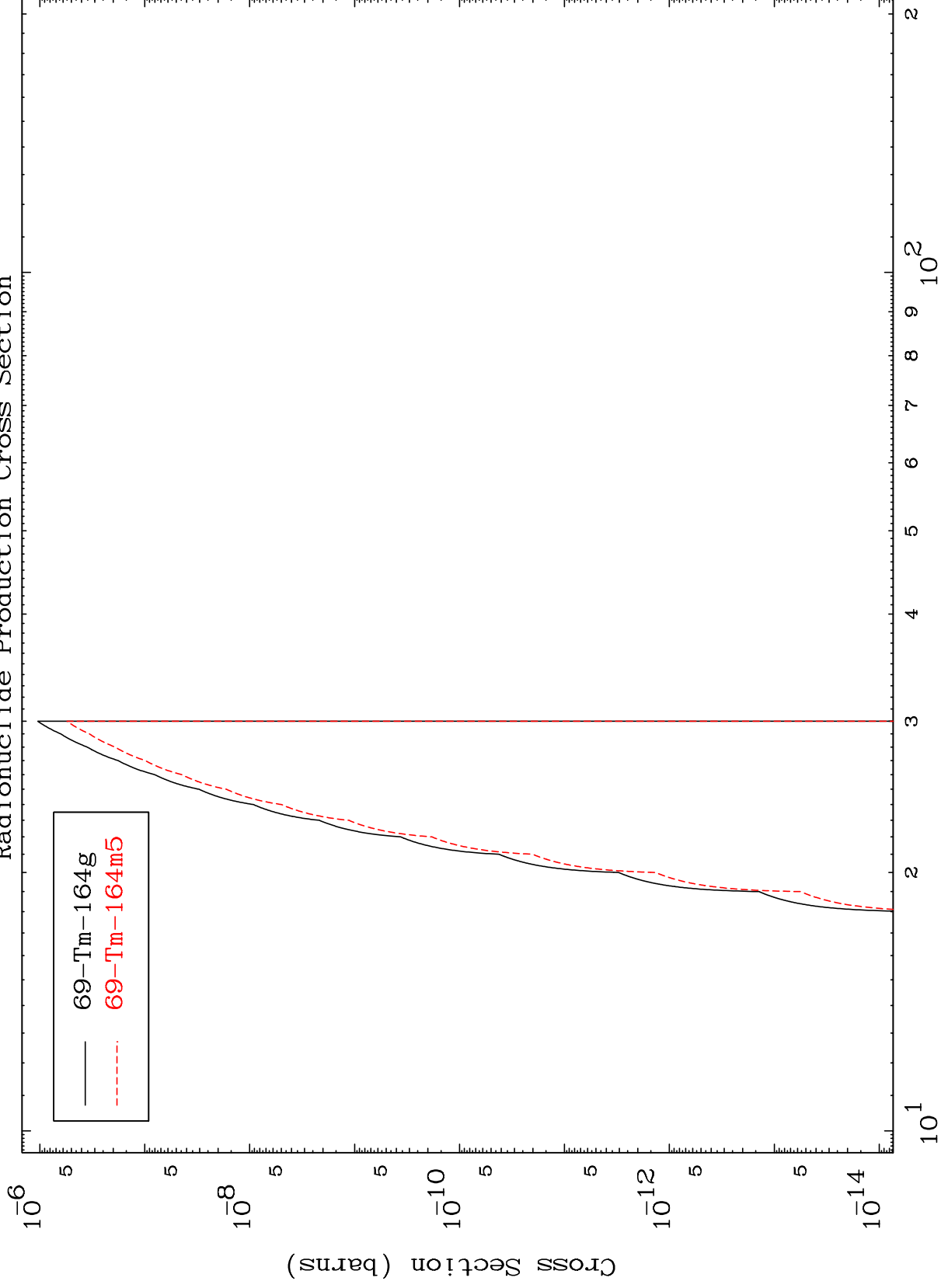


MAT 7101

(n,p) t

71-Lu-167

Radionuclide Production Cross Section



71-Lu-167

63