

Program EVALPLOT  
(Version 2018-1)

by

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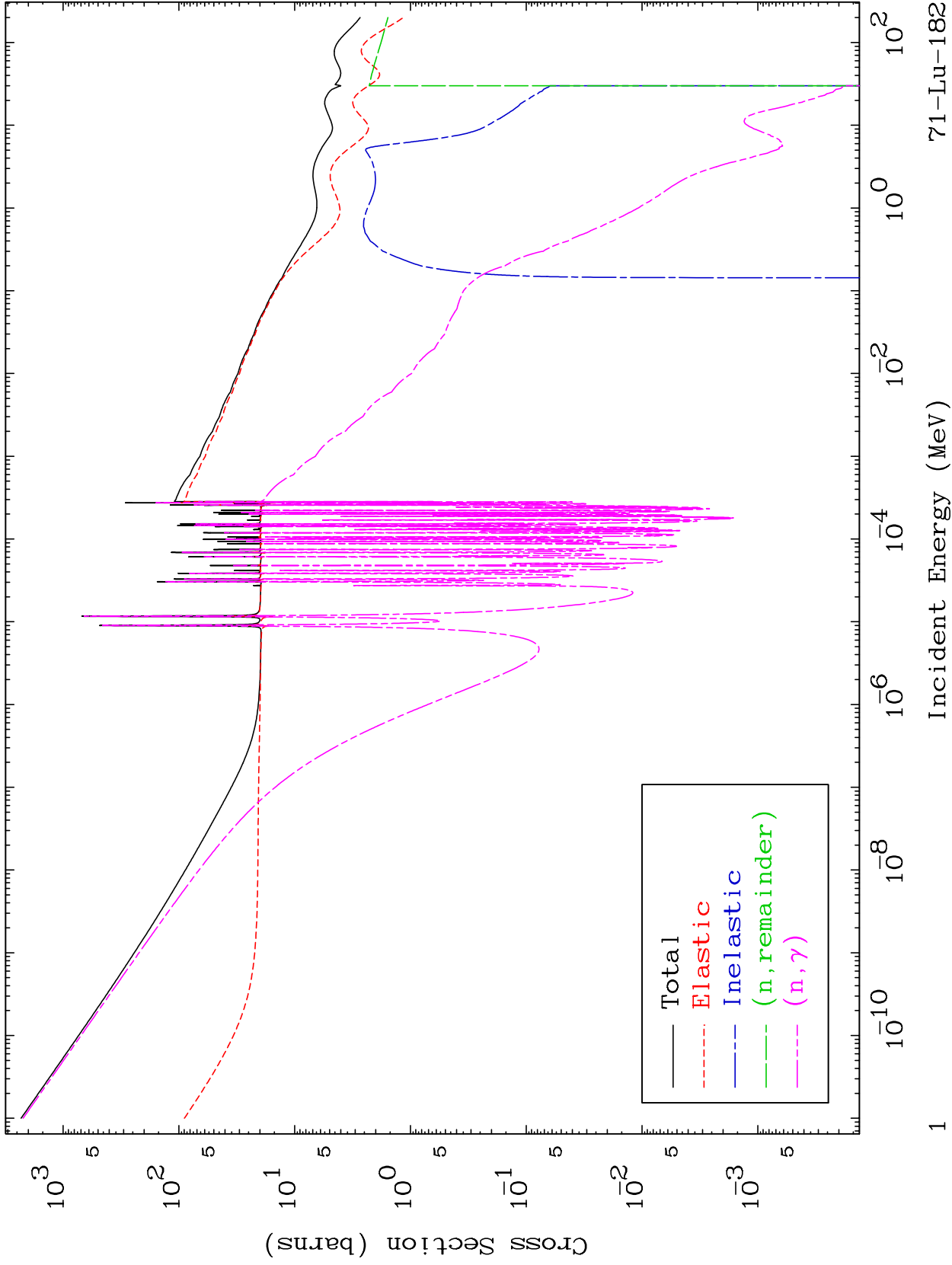
Press Mouse Button to Start

MAT 7146

Major

293 Kelvin Cross Sections

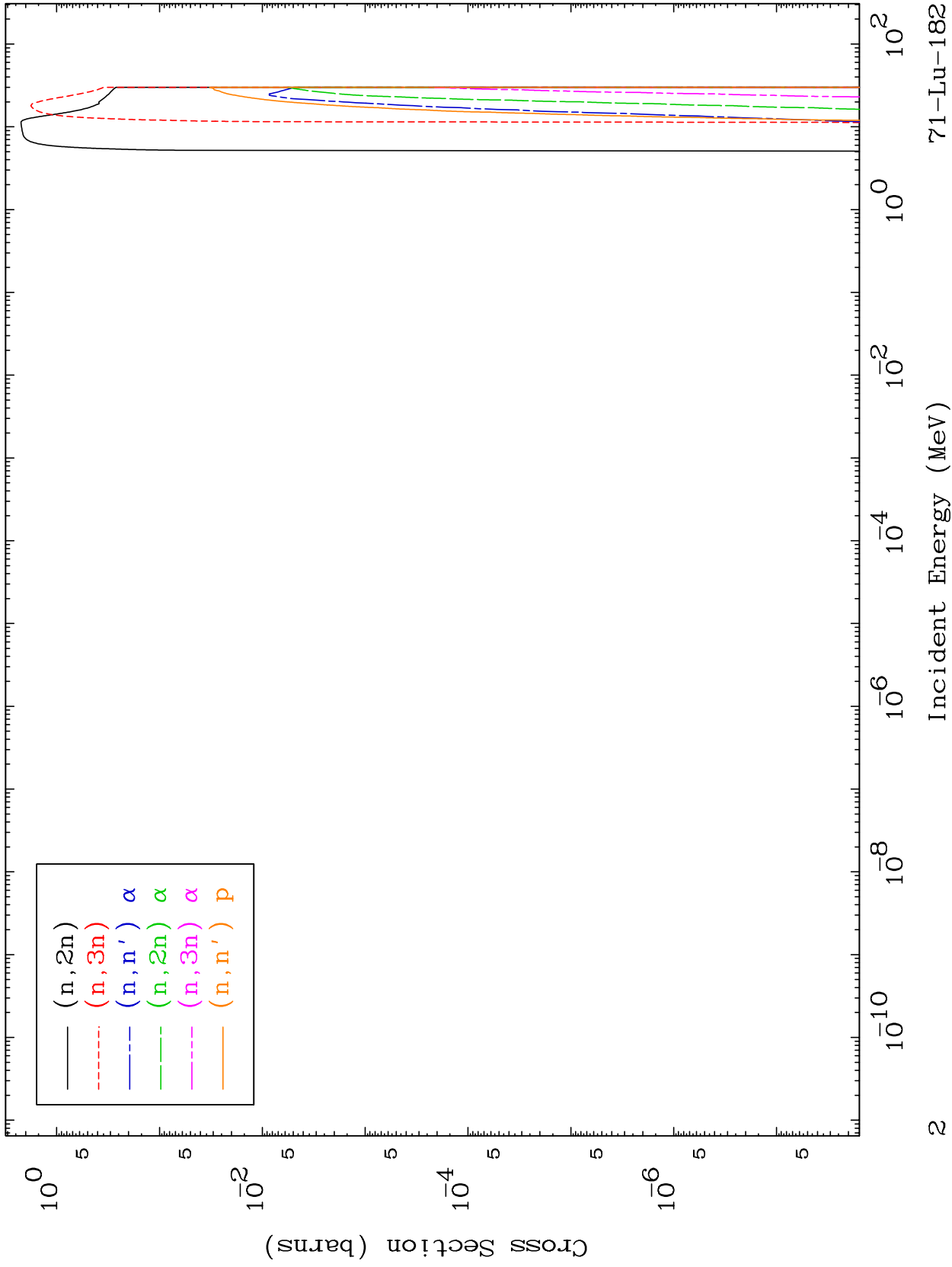
71-Lu-182

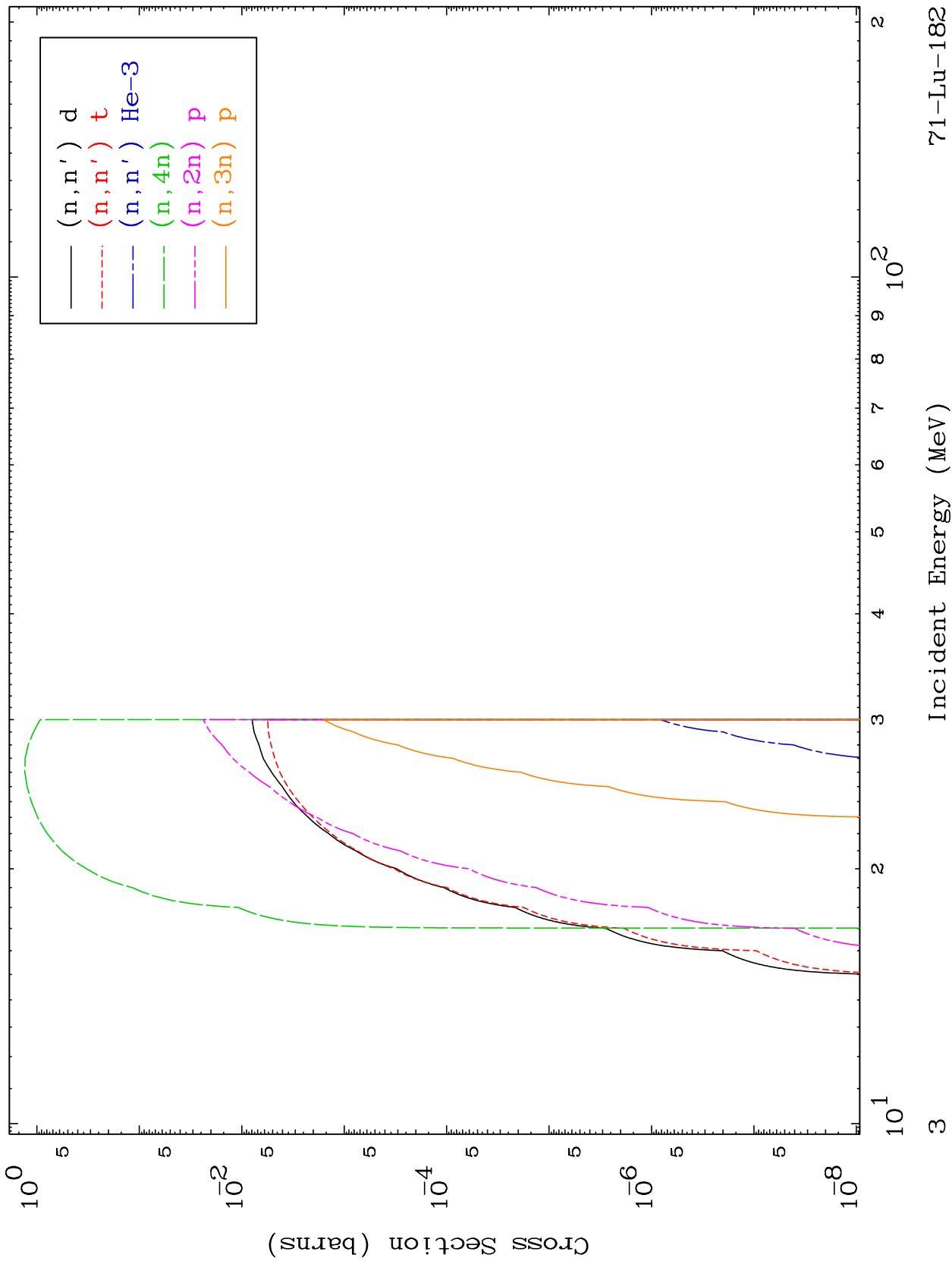


MAT 7146

Neutron Production  
293 Kelvin Cross Sections

71-Lu-182

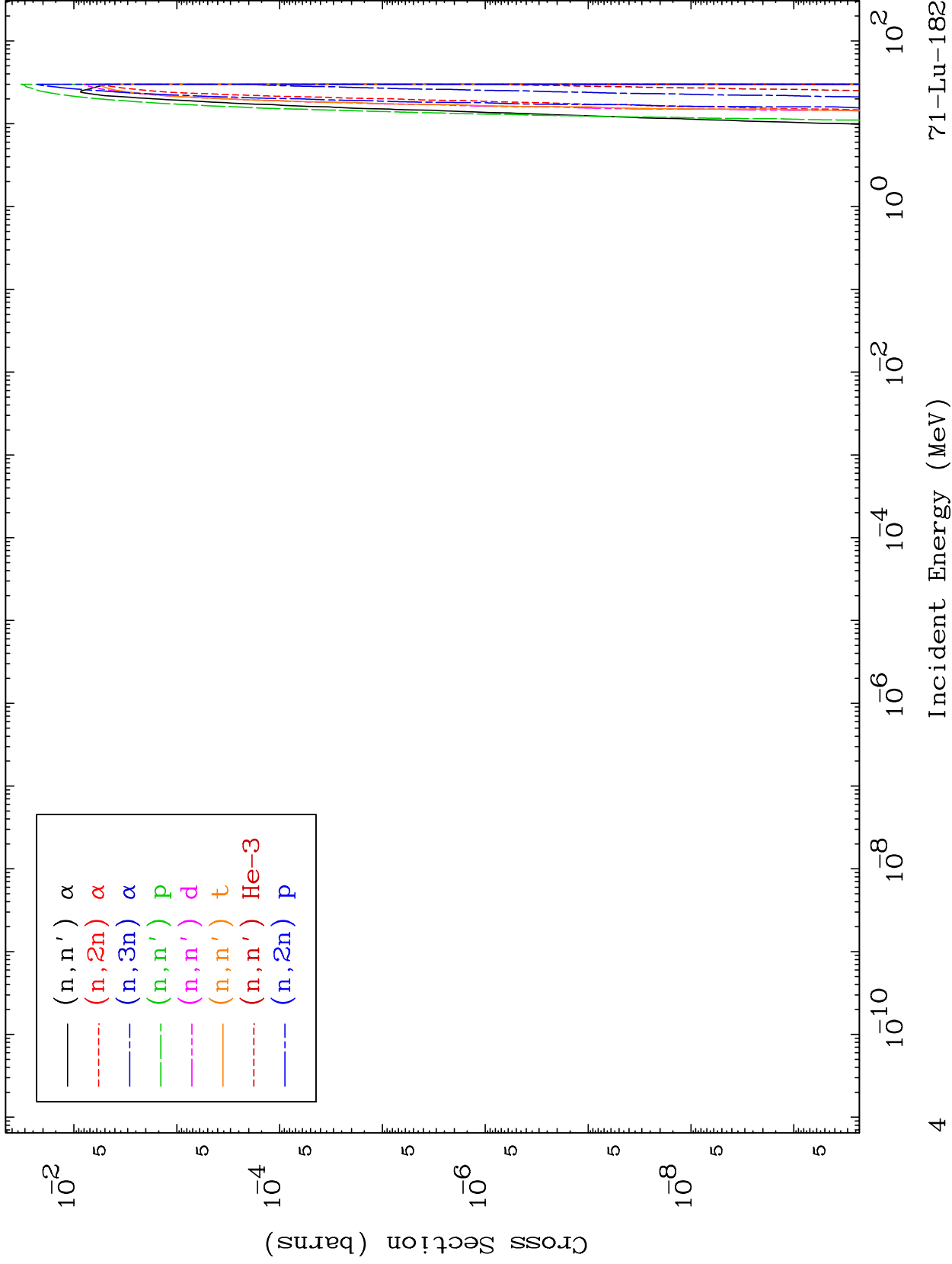




MAT 7146

Charged Particle  
293 Kelvin Cross Sections

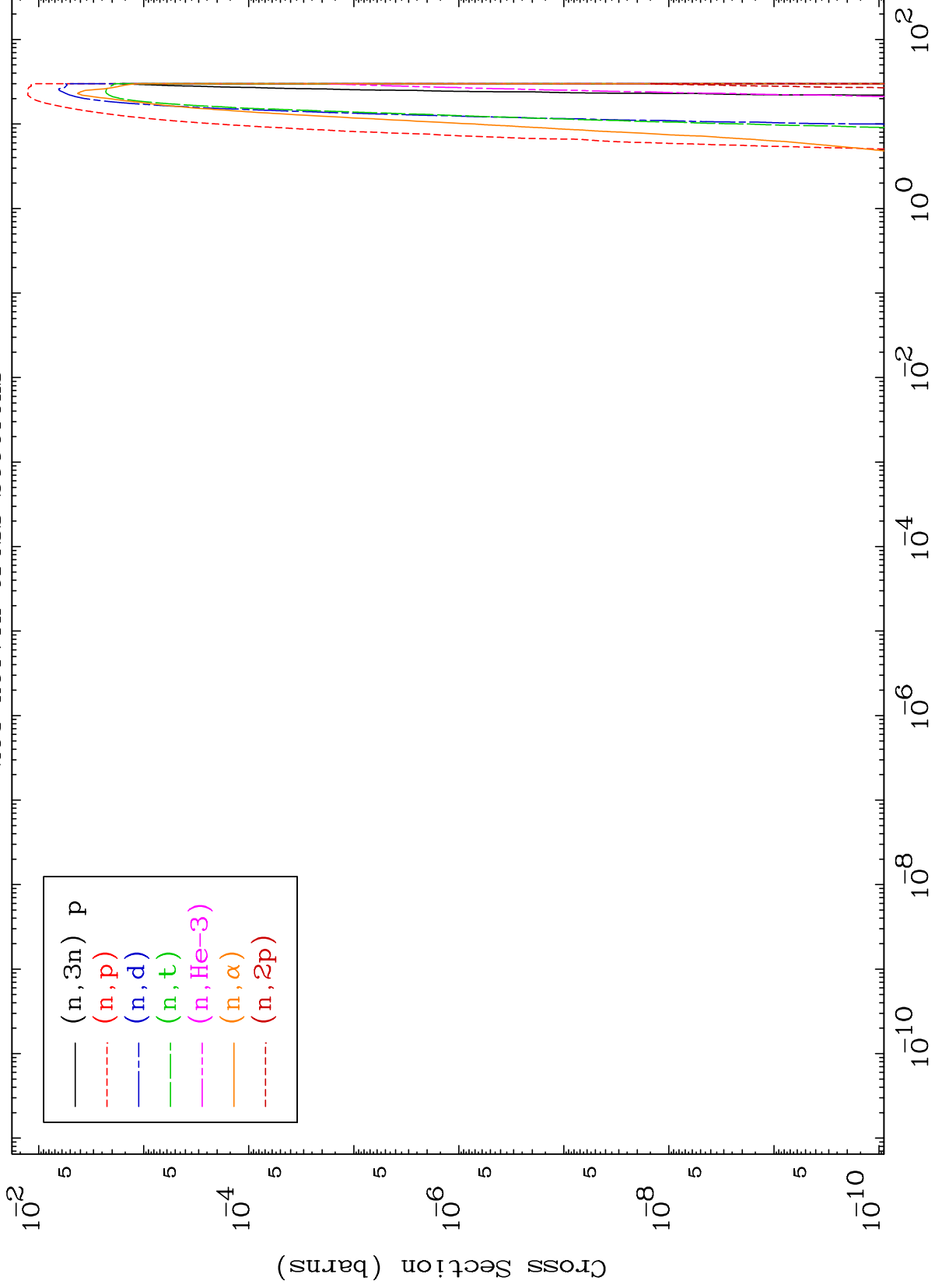
71-Lu-182



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Charged Particle  
293 Kelvin Cross Sections

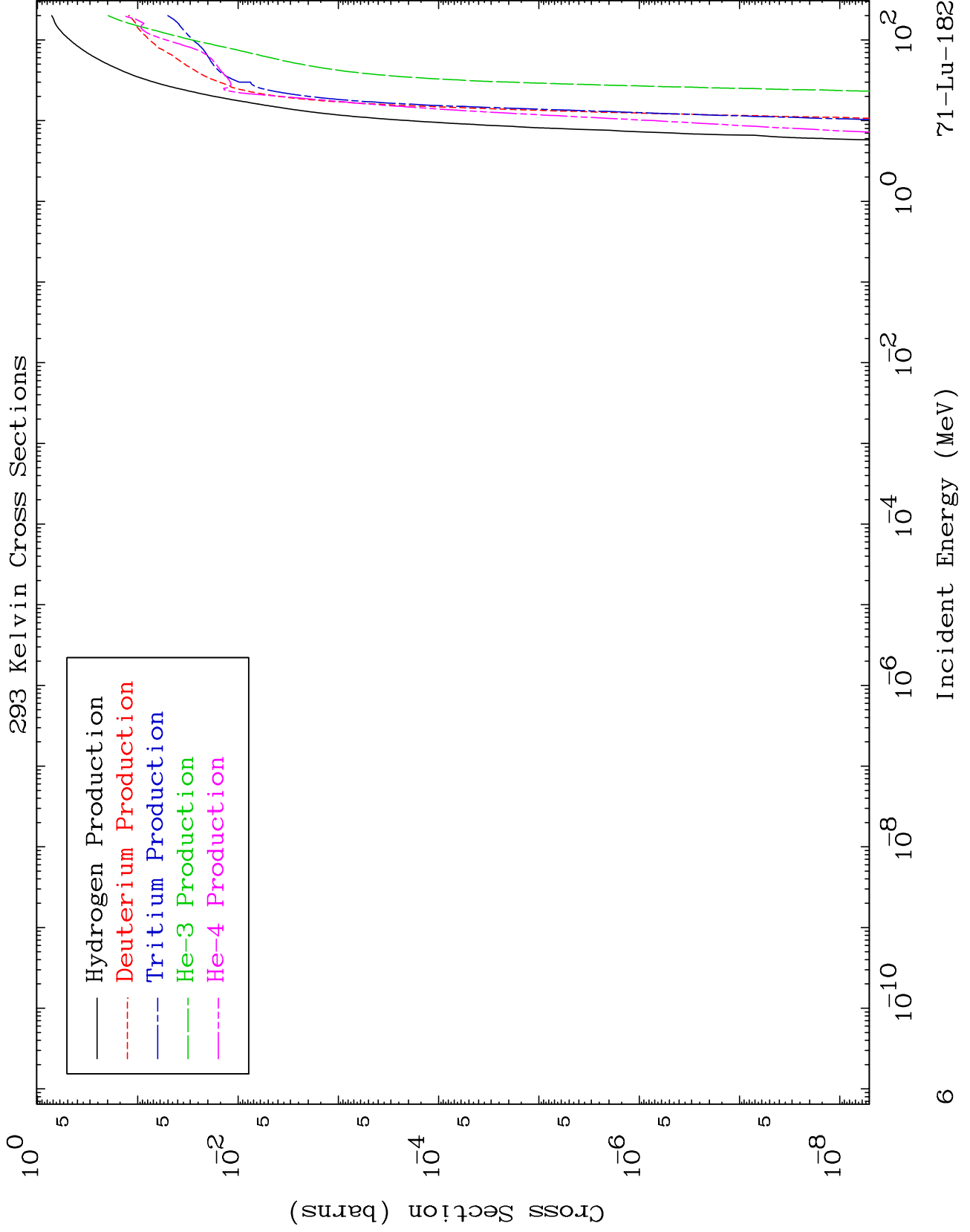
71-Lu-182



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Particle Production  
293 Kelvin Cross Sections

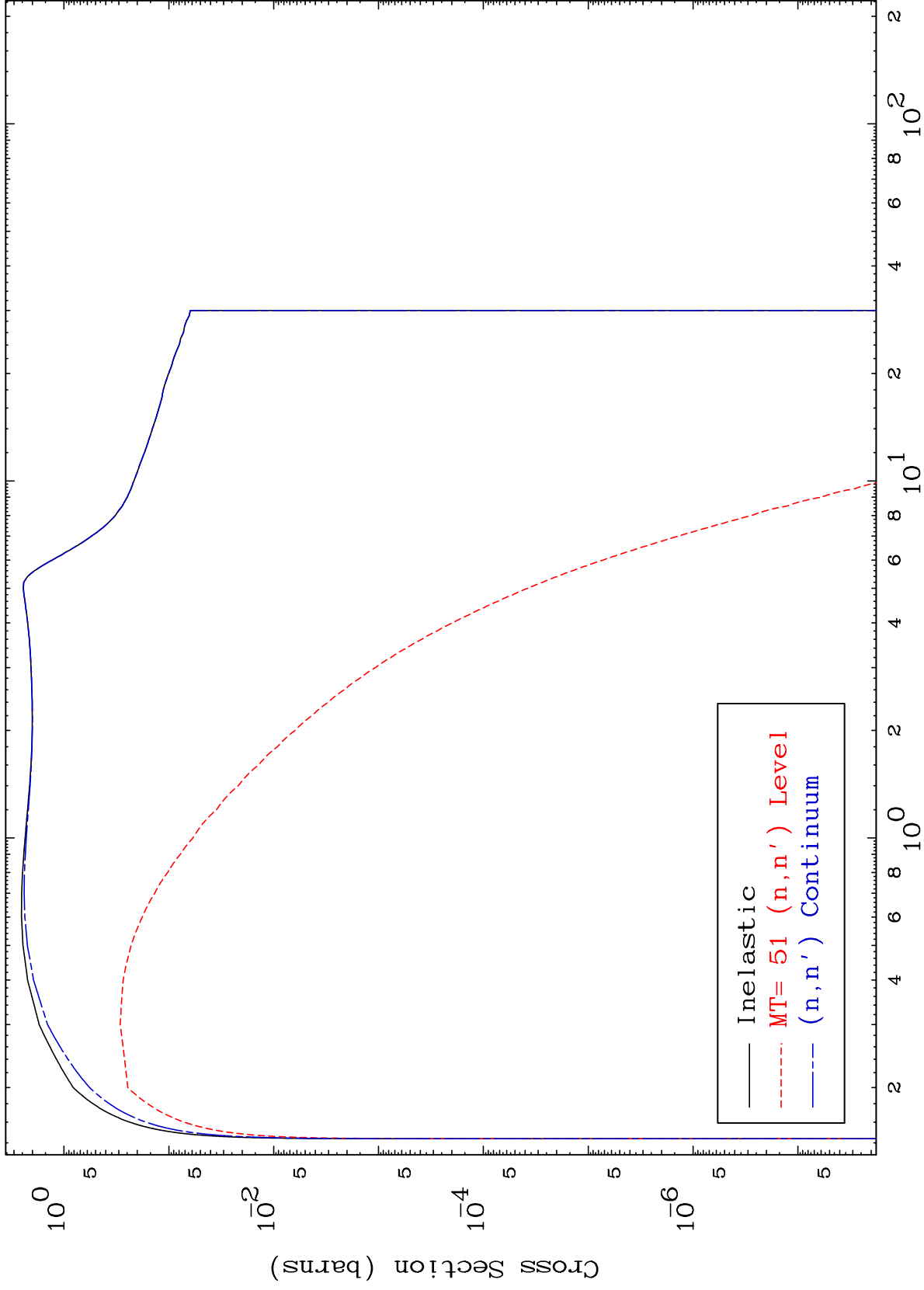
71-Lu-182



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(n,n') Level  
293 Kelvin Cross Sections

71-Lu-182

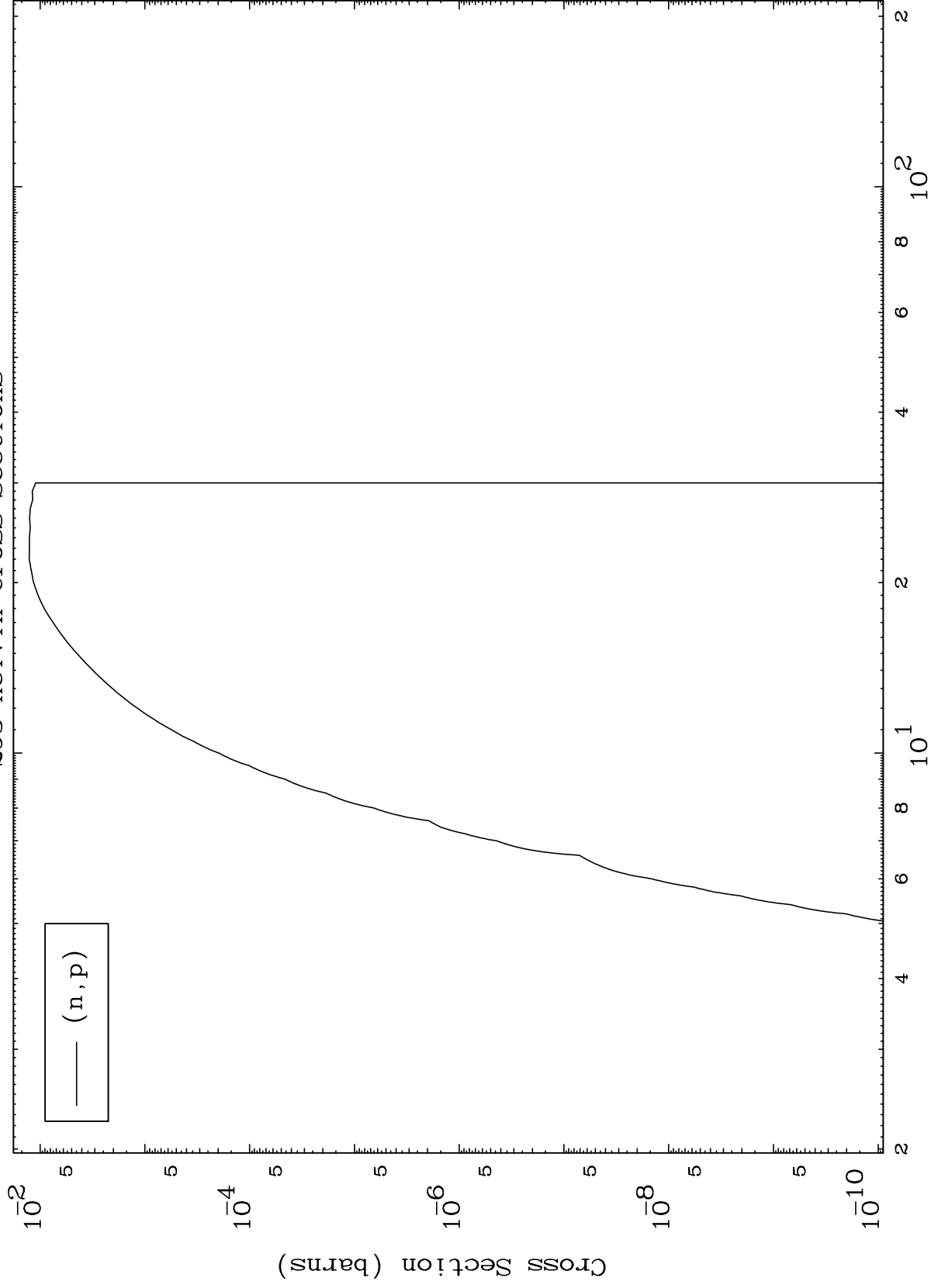




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(n,p) Levels  
293 Kelvin Cross Sections

71-Lu-182



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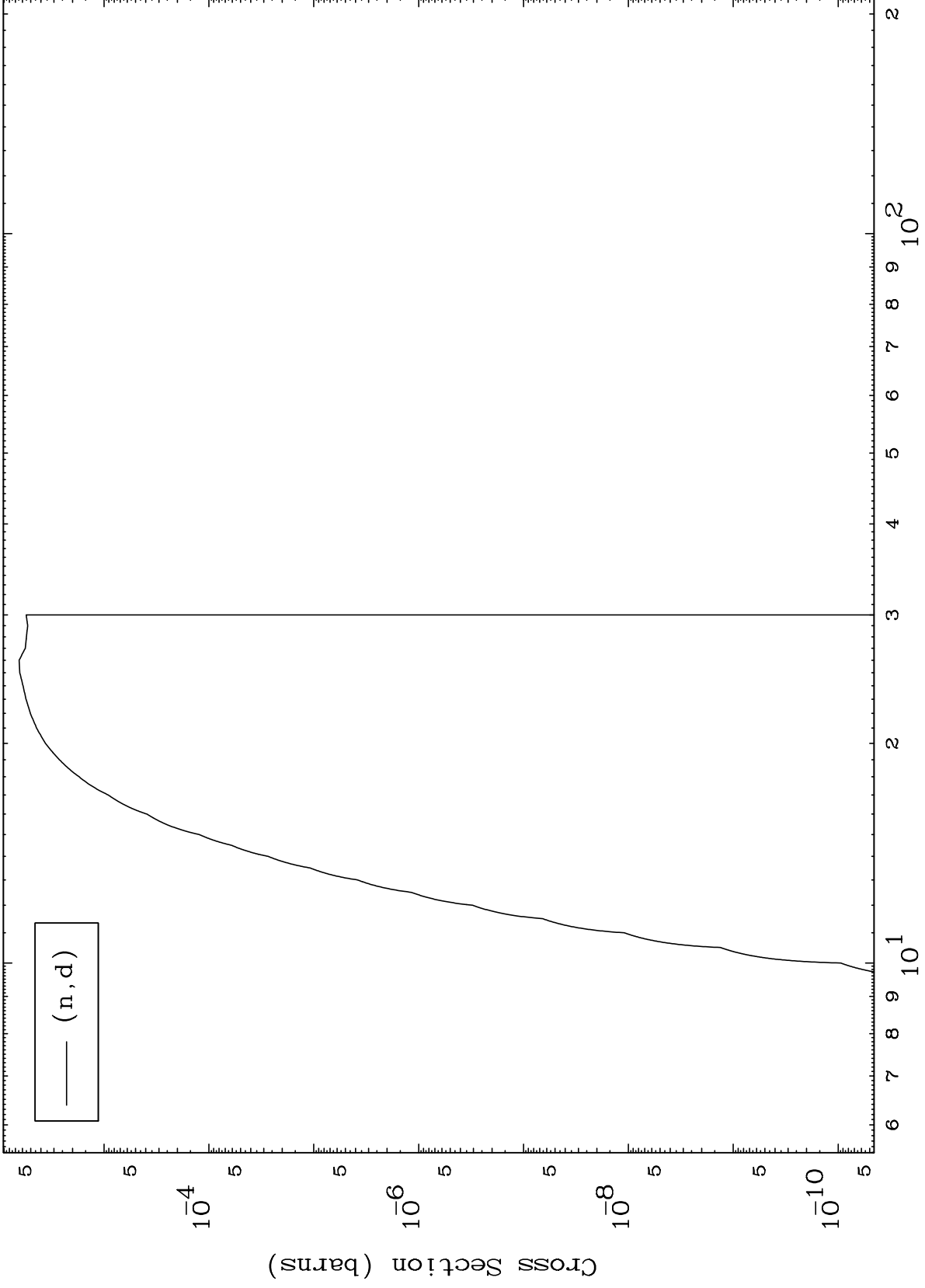
Incident Energy (MeV)

71-Lu-182

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(n,d) Levels  
293 Kelvin Cross Sections

71-Lu-182



9

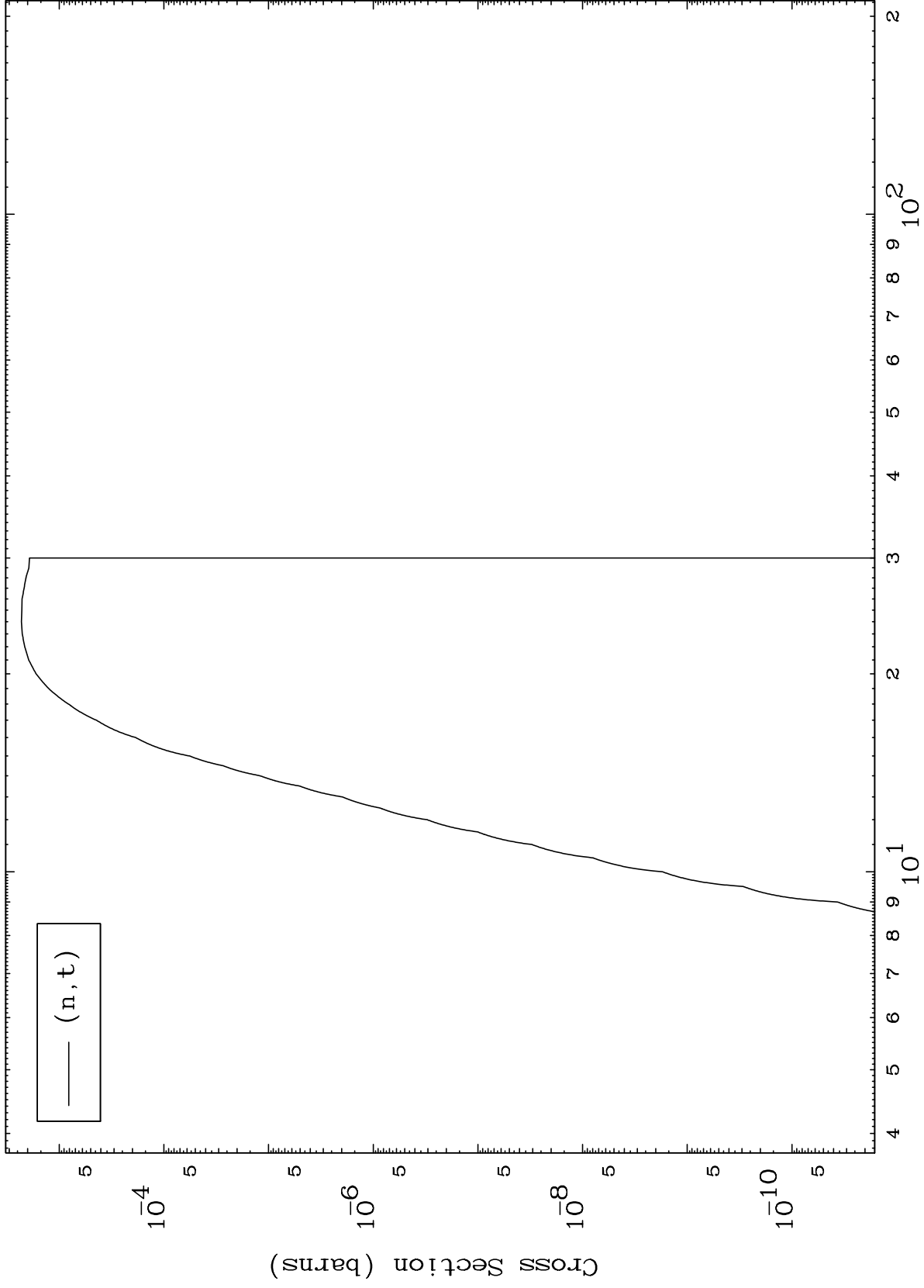
Incident Energy (MeV)

71-Lu-182

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(n,t) Levels  
293 Kelvin Cross Sections

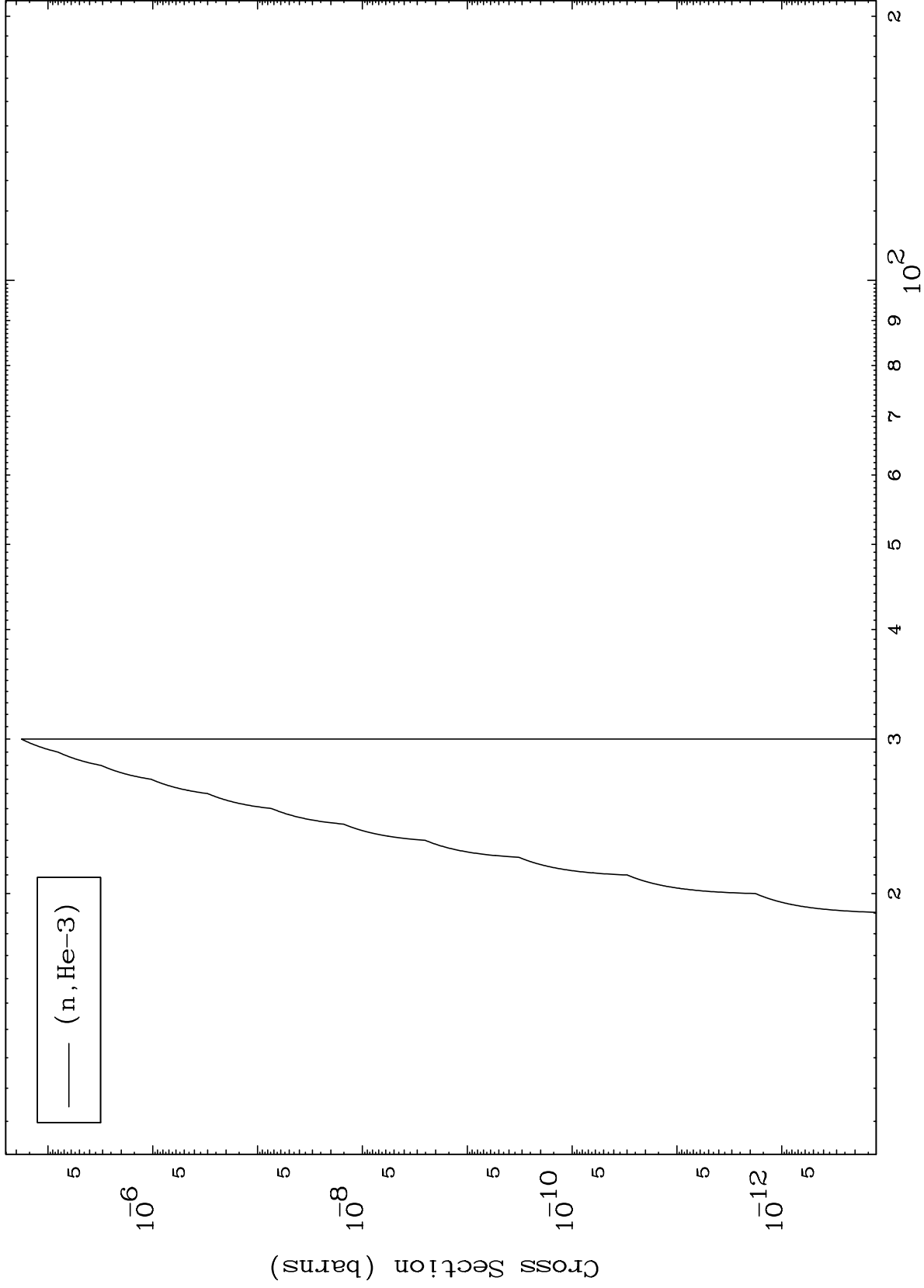
71-Lu-182



10

Incident Energy (MeV)

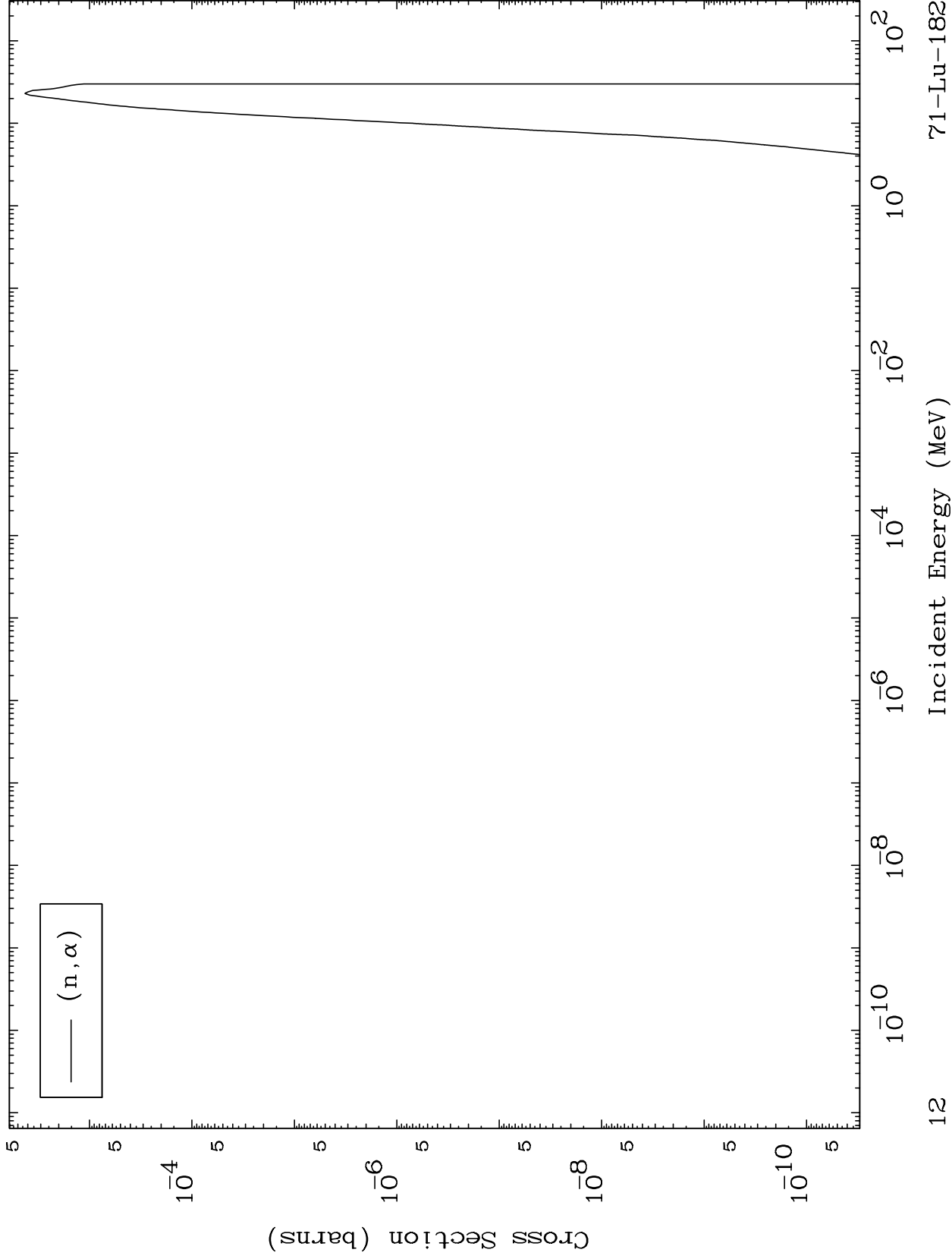
71-Lu-182



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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

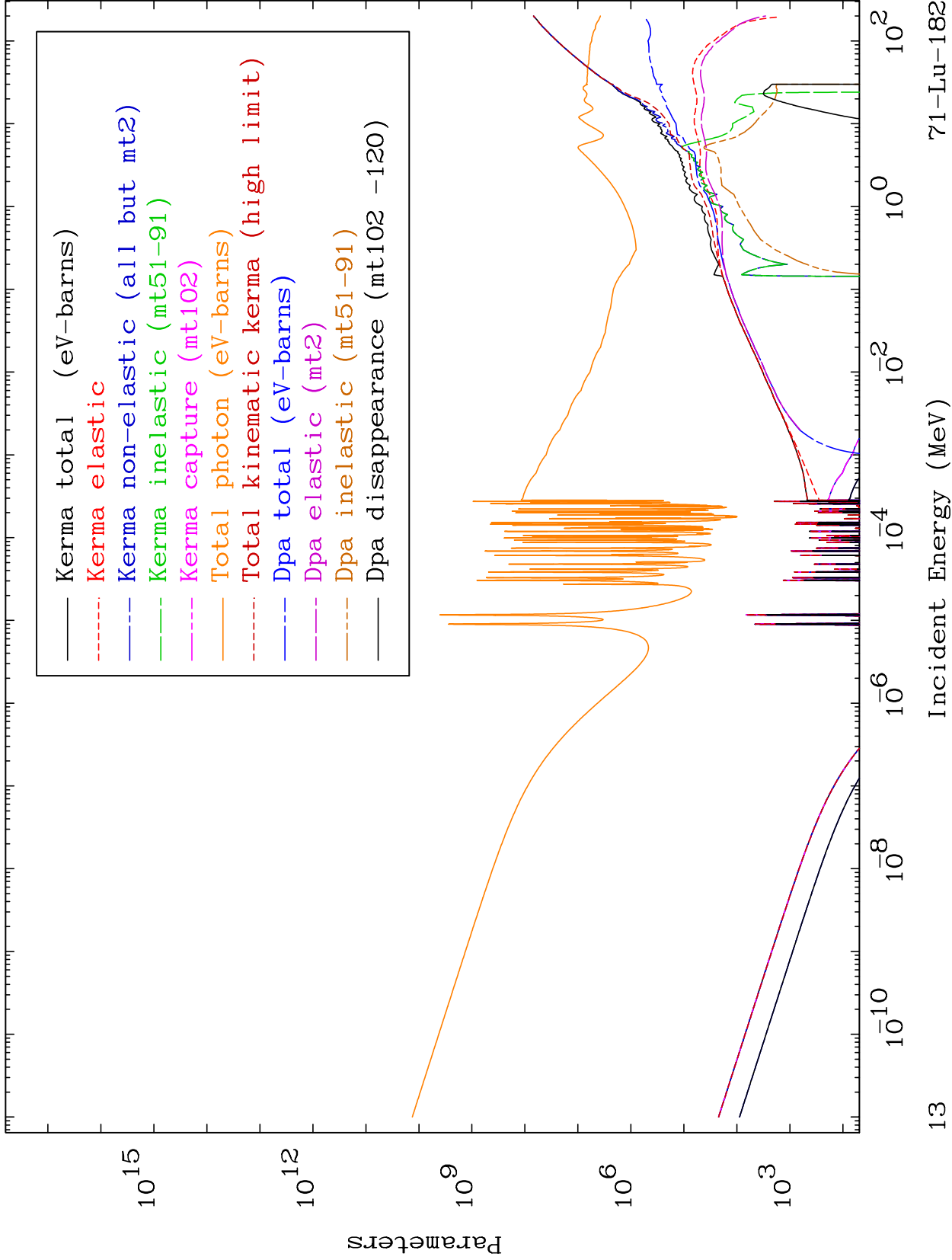
71-Lu-182



12

Incident Energy (MeV)

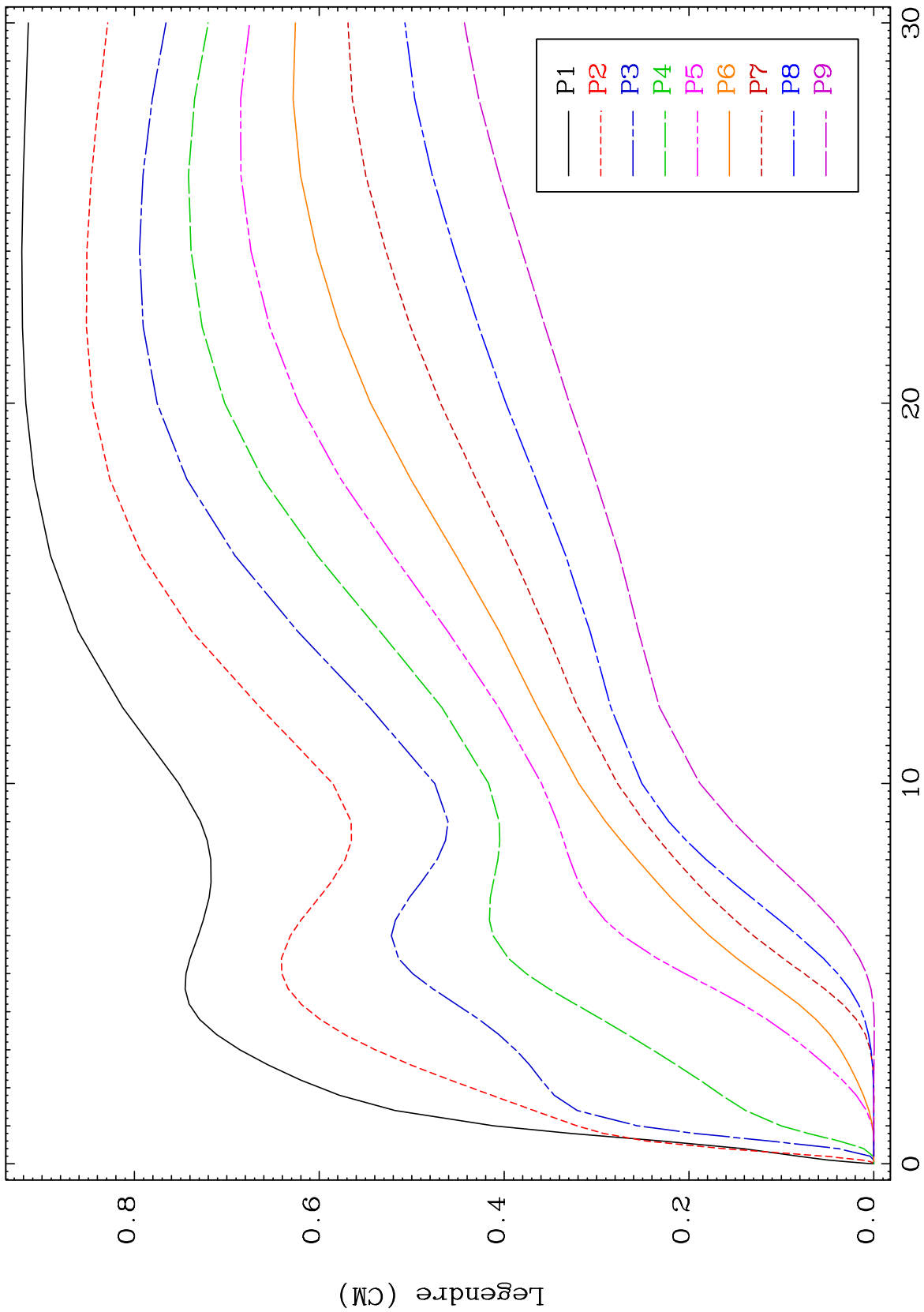
71-Lu-182



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71-Lu-182

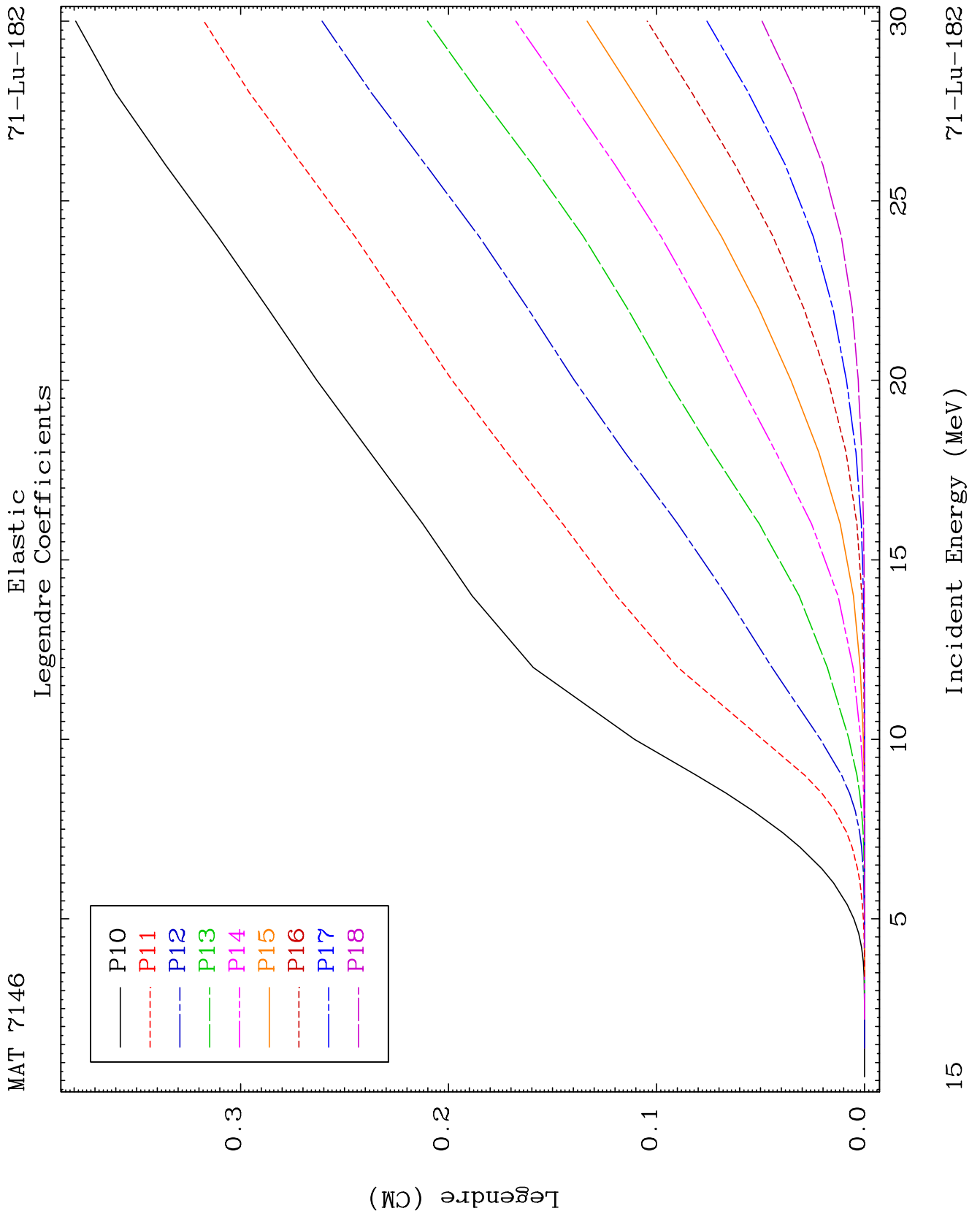
Elastic Legendre Coefficients



14

Incident Energy (MeV)

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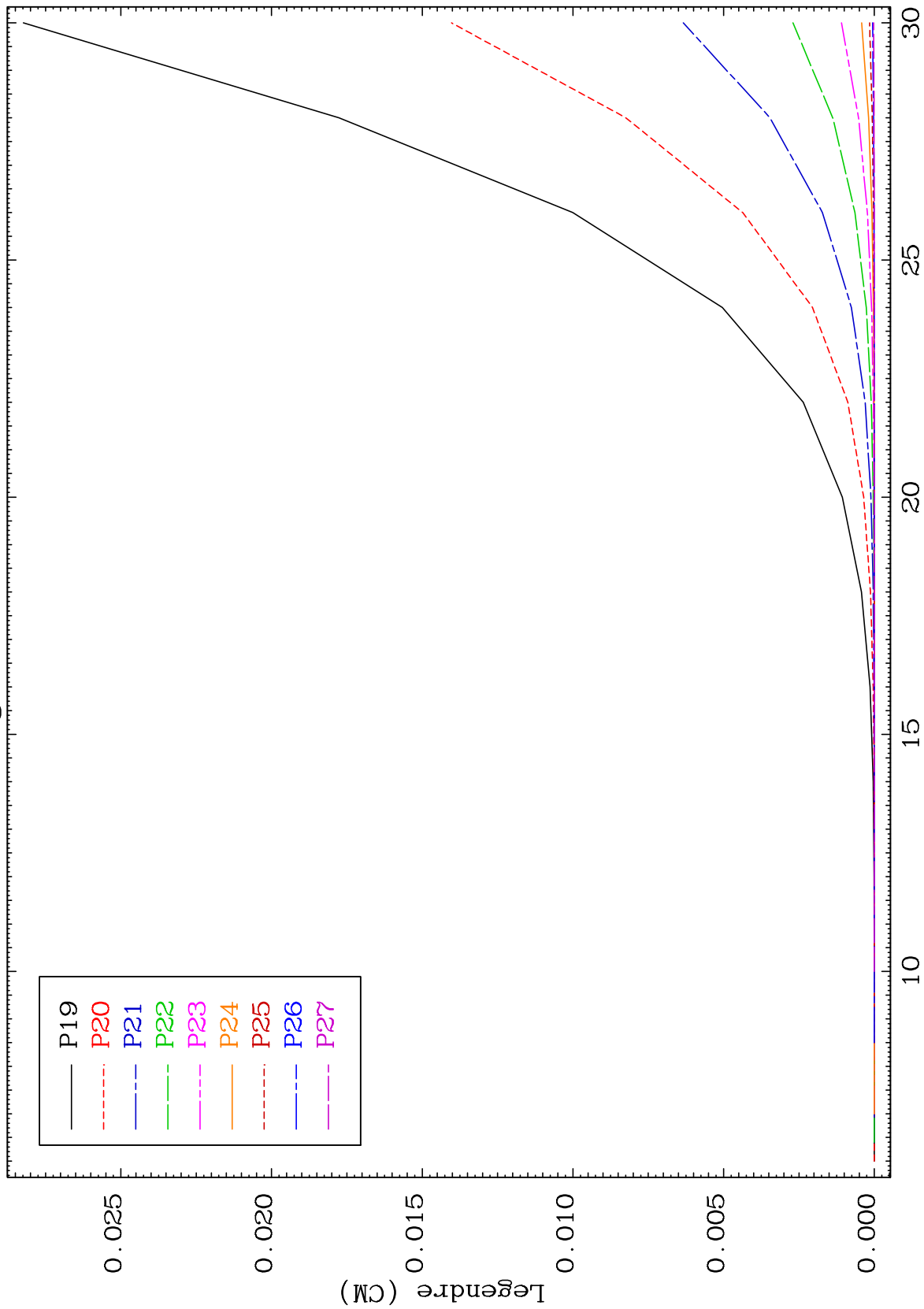




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### Elastic Legendre Coefficients

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16

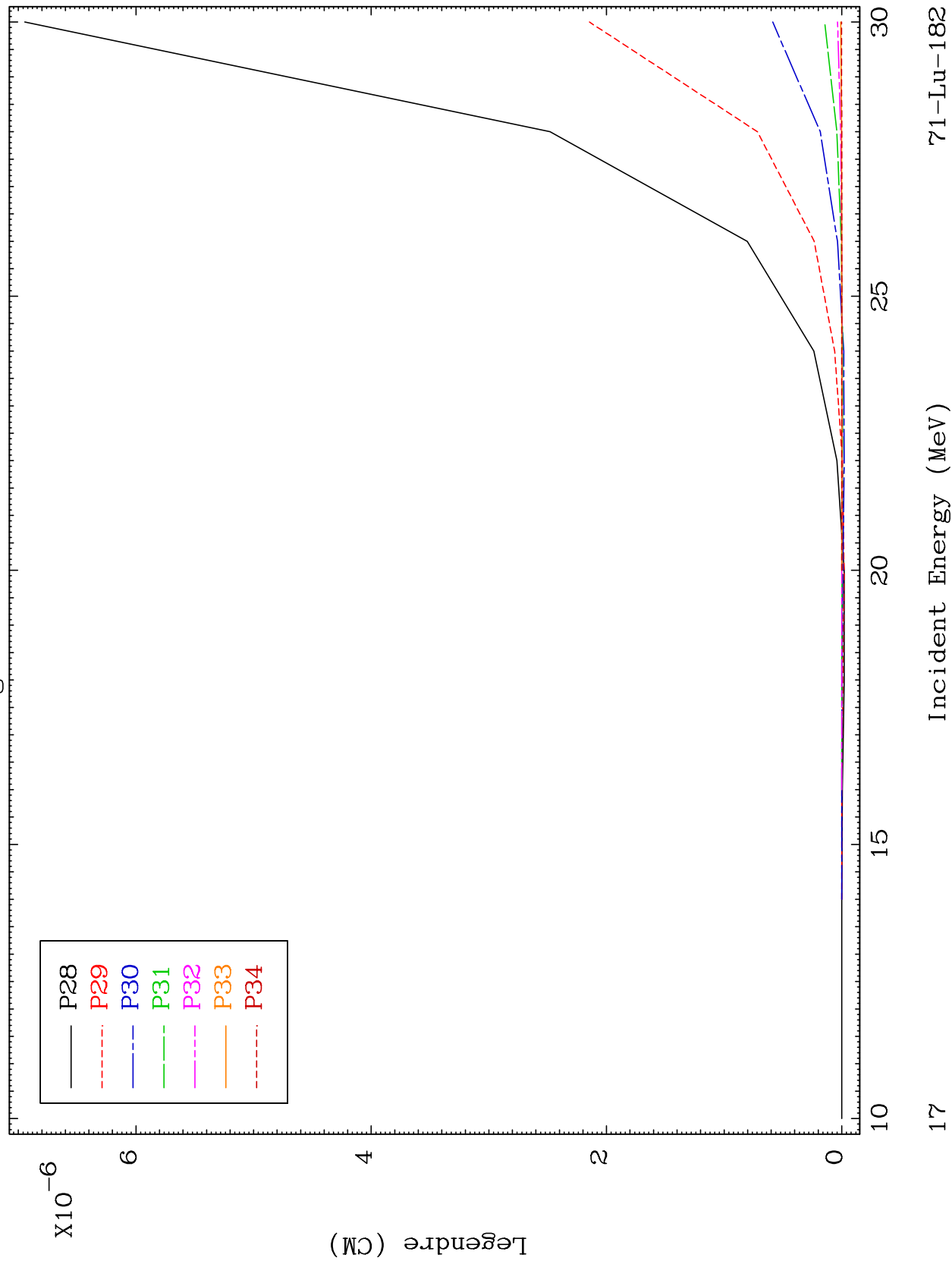
Incident Energy (MeV)

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Elastic  
Legendre Coefficients

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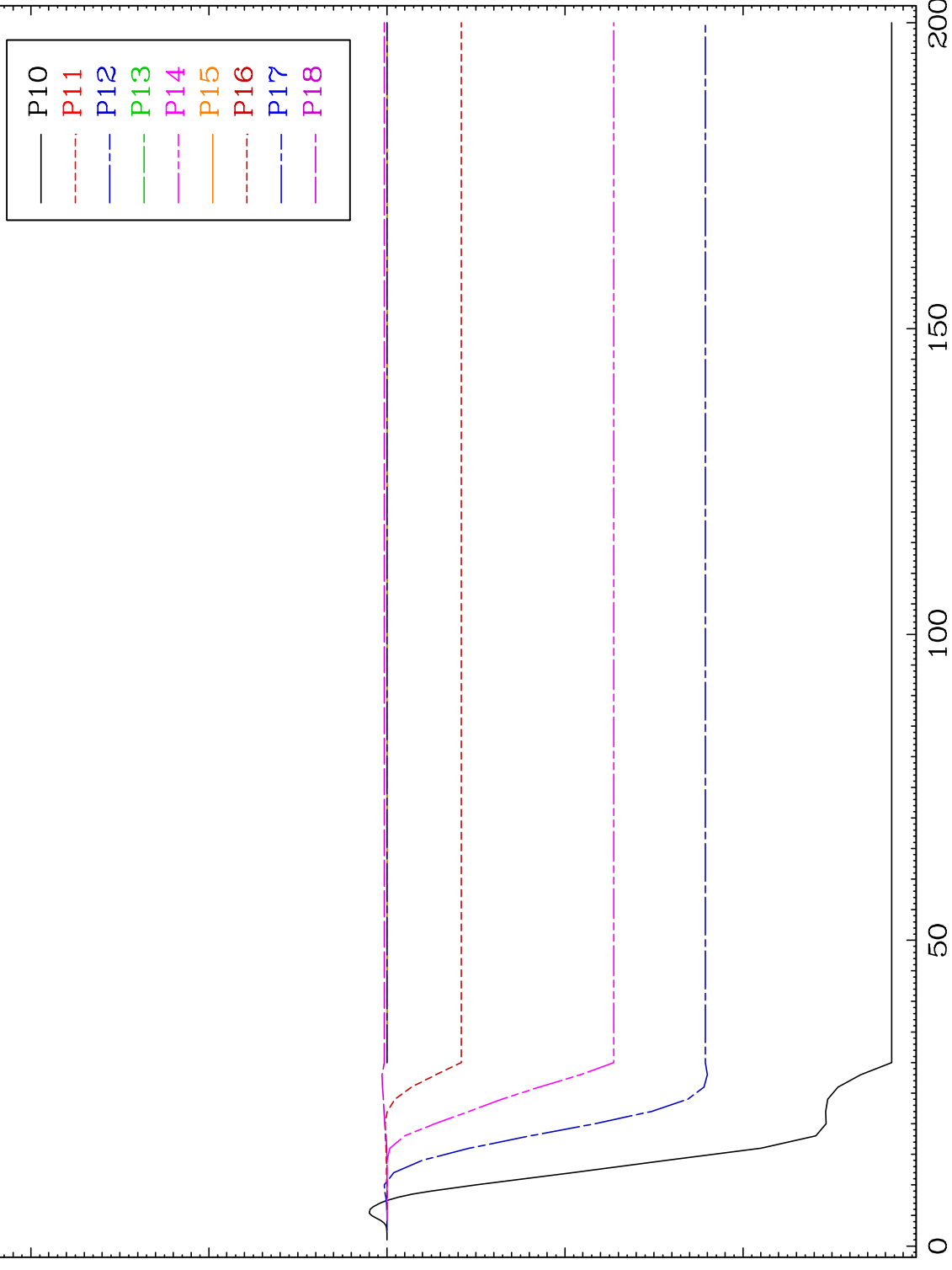
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MT= 51 (n,n') Level  
Legendre Coefficients

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$\times 10^{-44}$

Legendre (CM)



0

50

100

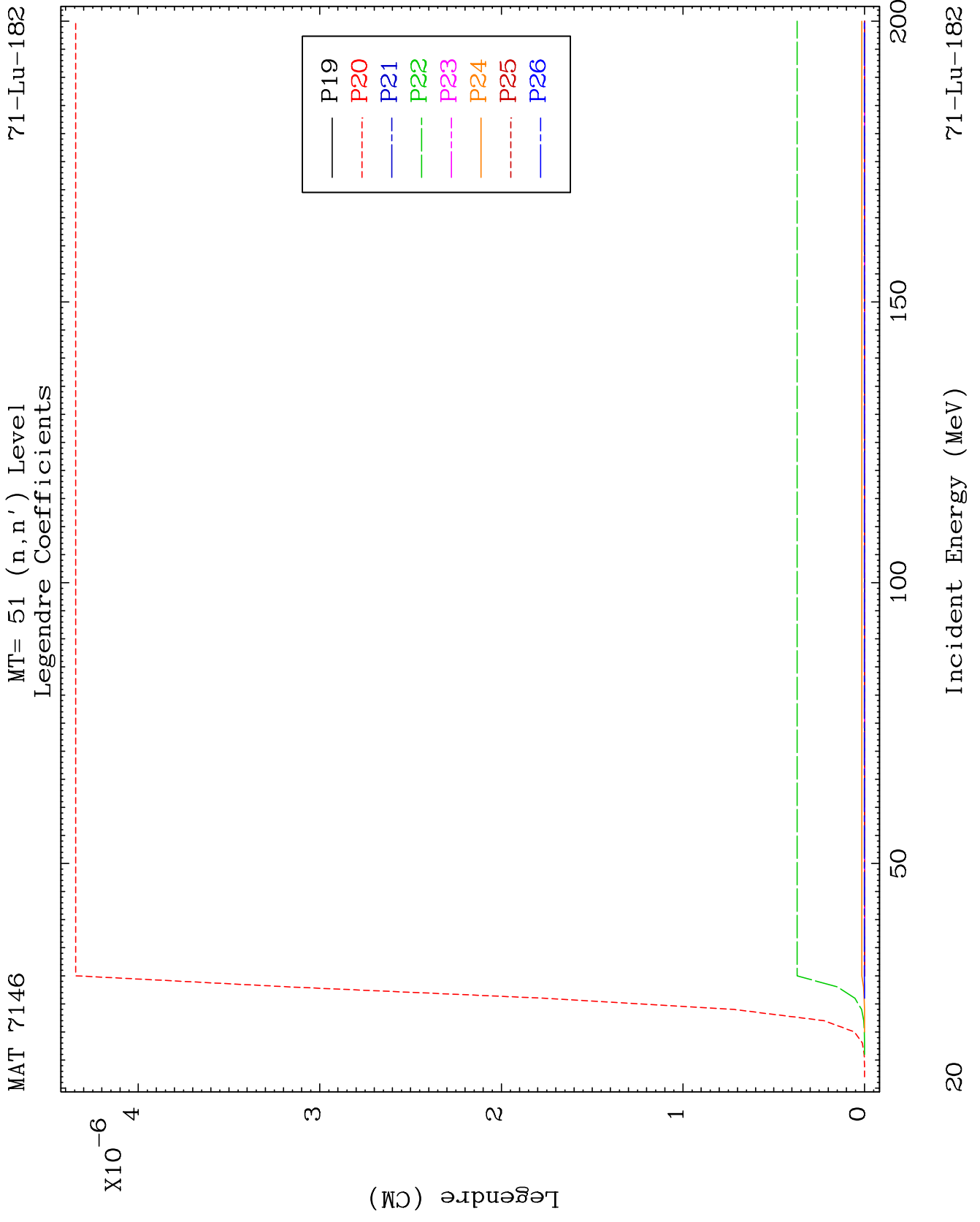
150

200

Incident Energy (MeV)

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19



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Fission

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Radionuclide Production Cross Section

