

Program EVALPLOT  
(Version 2018-1)

by

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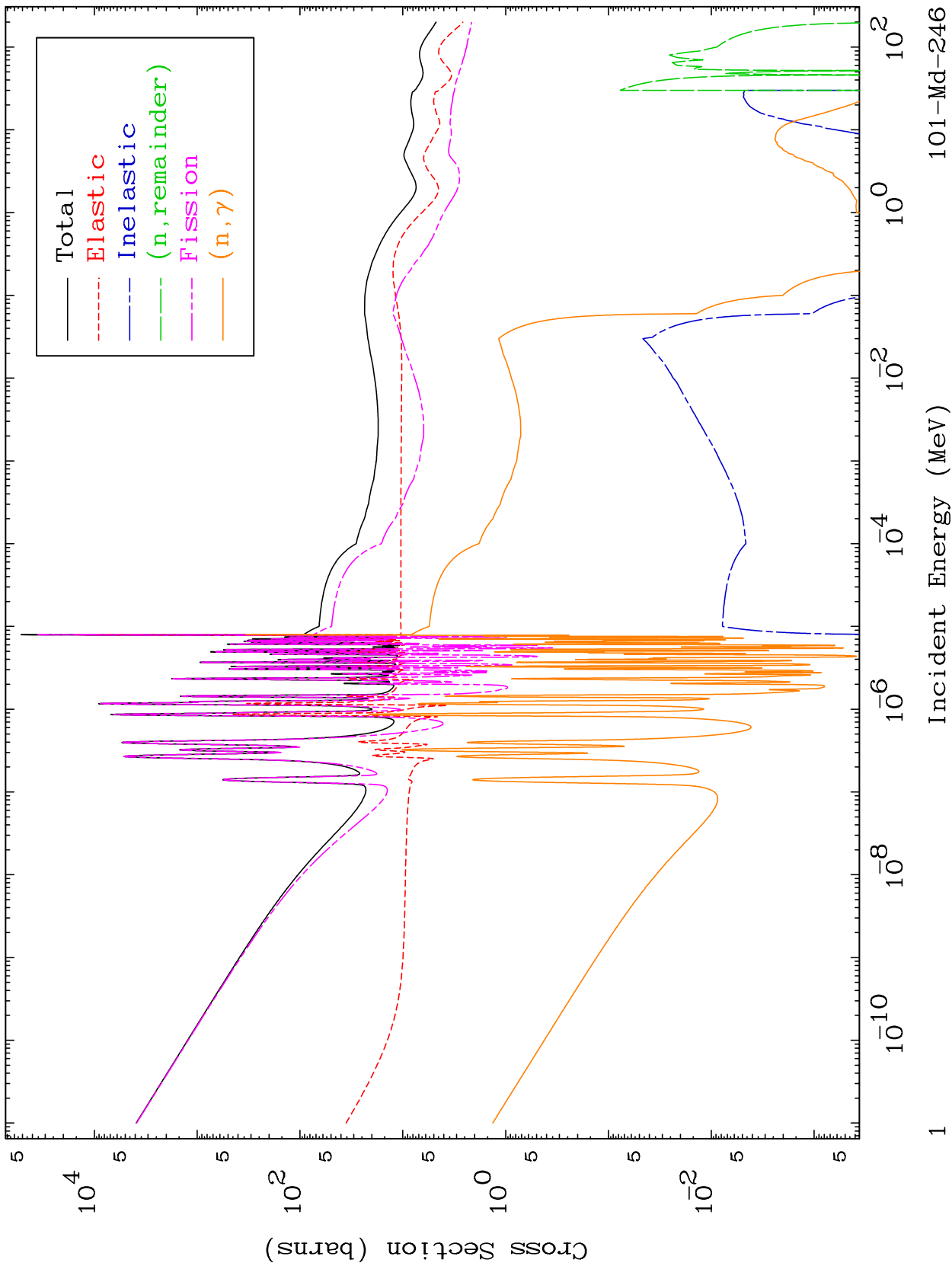
Press Mouse Button to Start

MAT 9952

Major

293 Kelvin Cross Sections

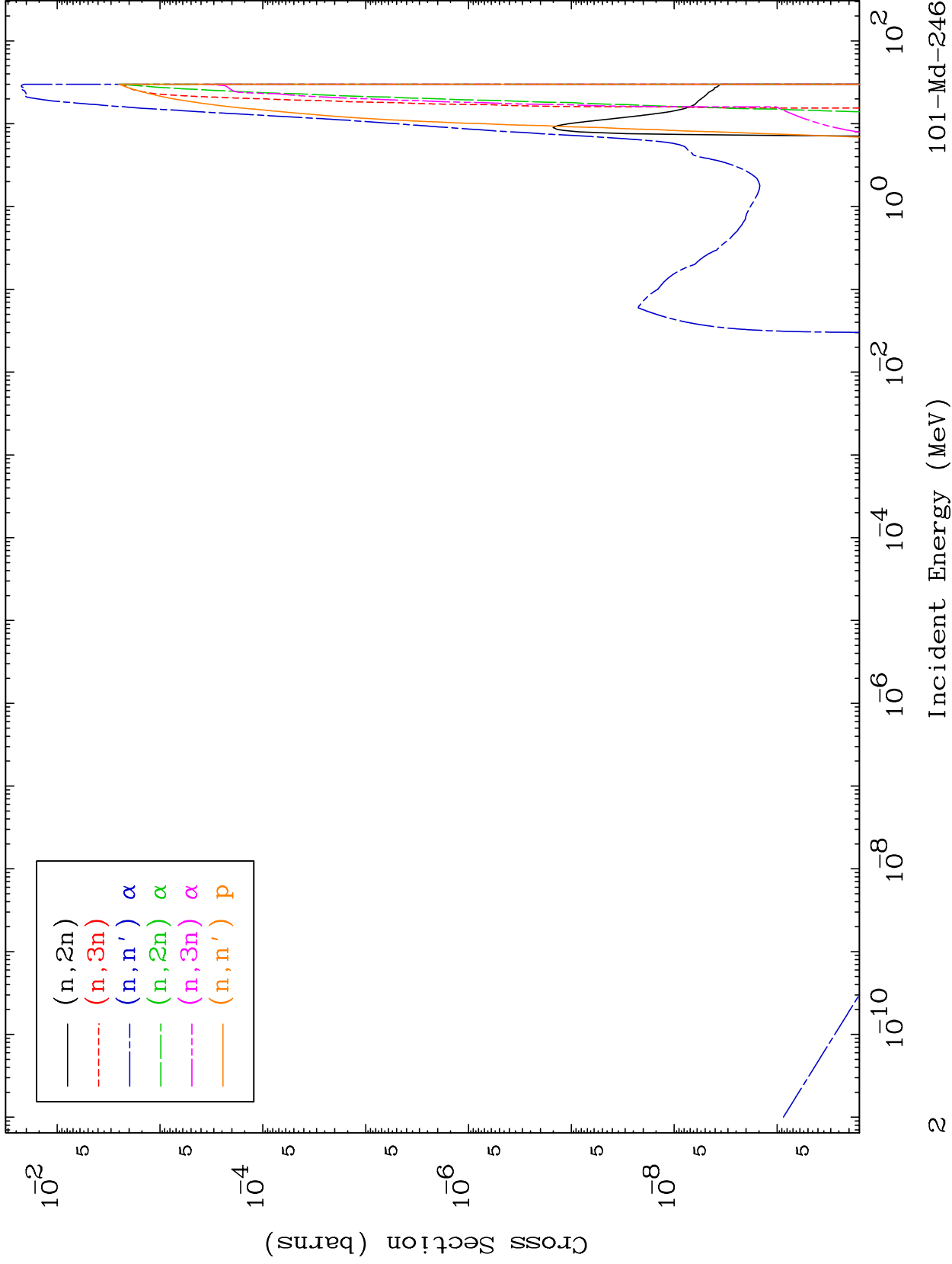
101-Md-246

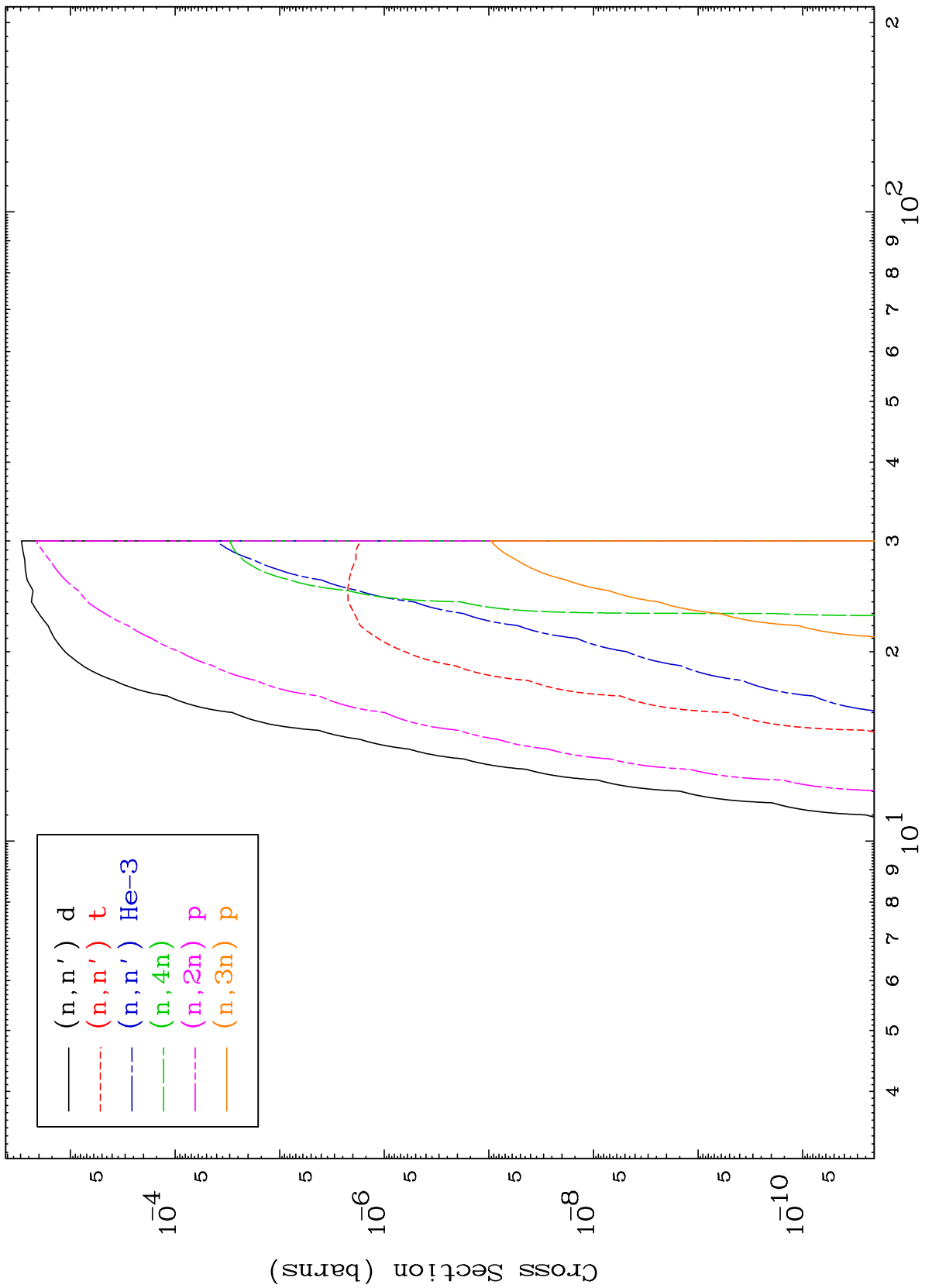


MAT 9952

Neutron Production  
293 Kelvin Cross Sections

101-Md-246

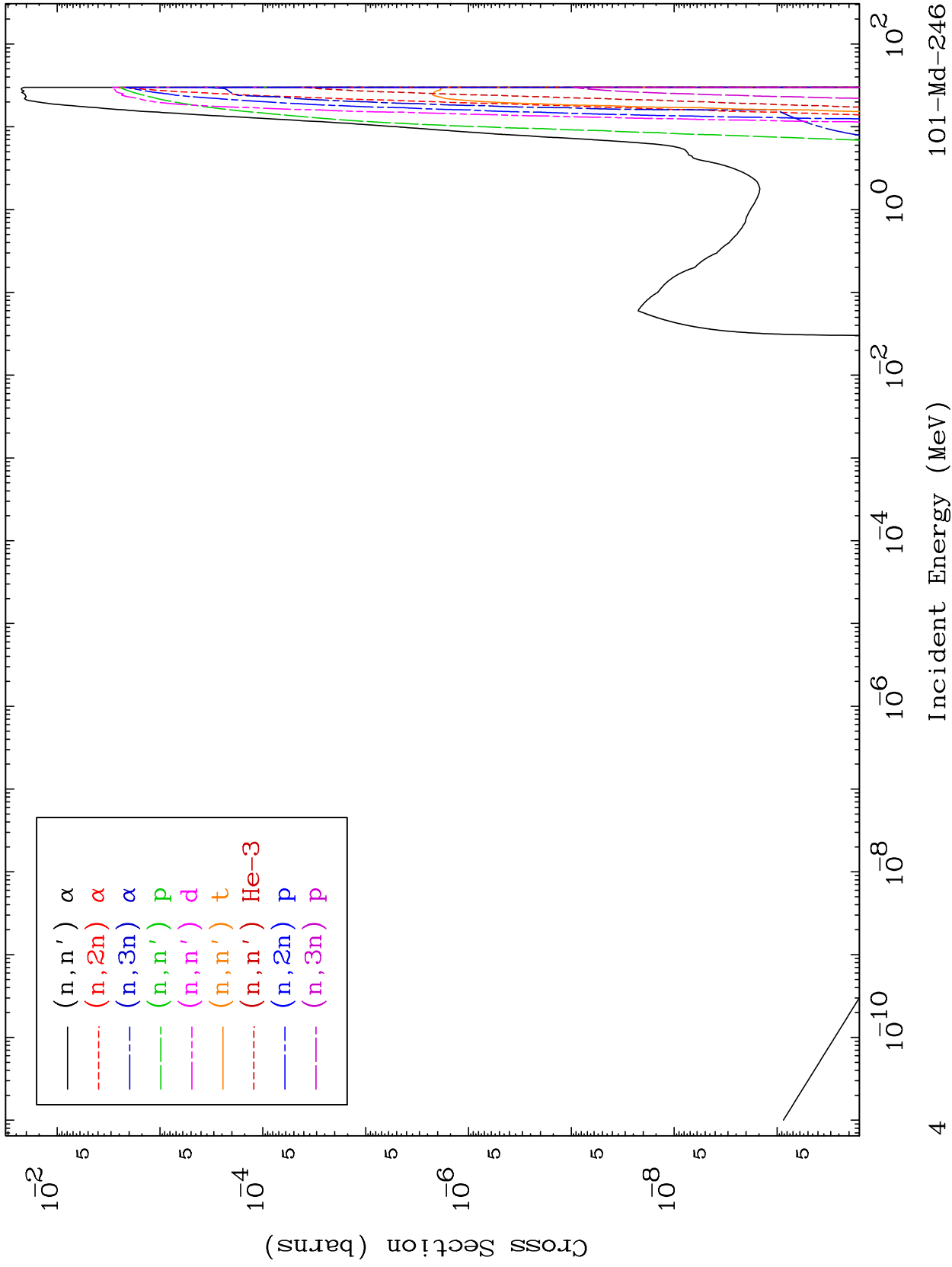




MAT 9952

Charged Particle  
293 Kelvin Cross Sections

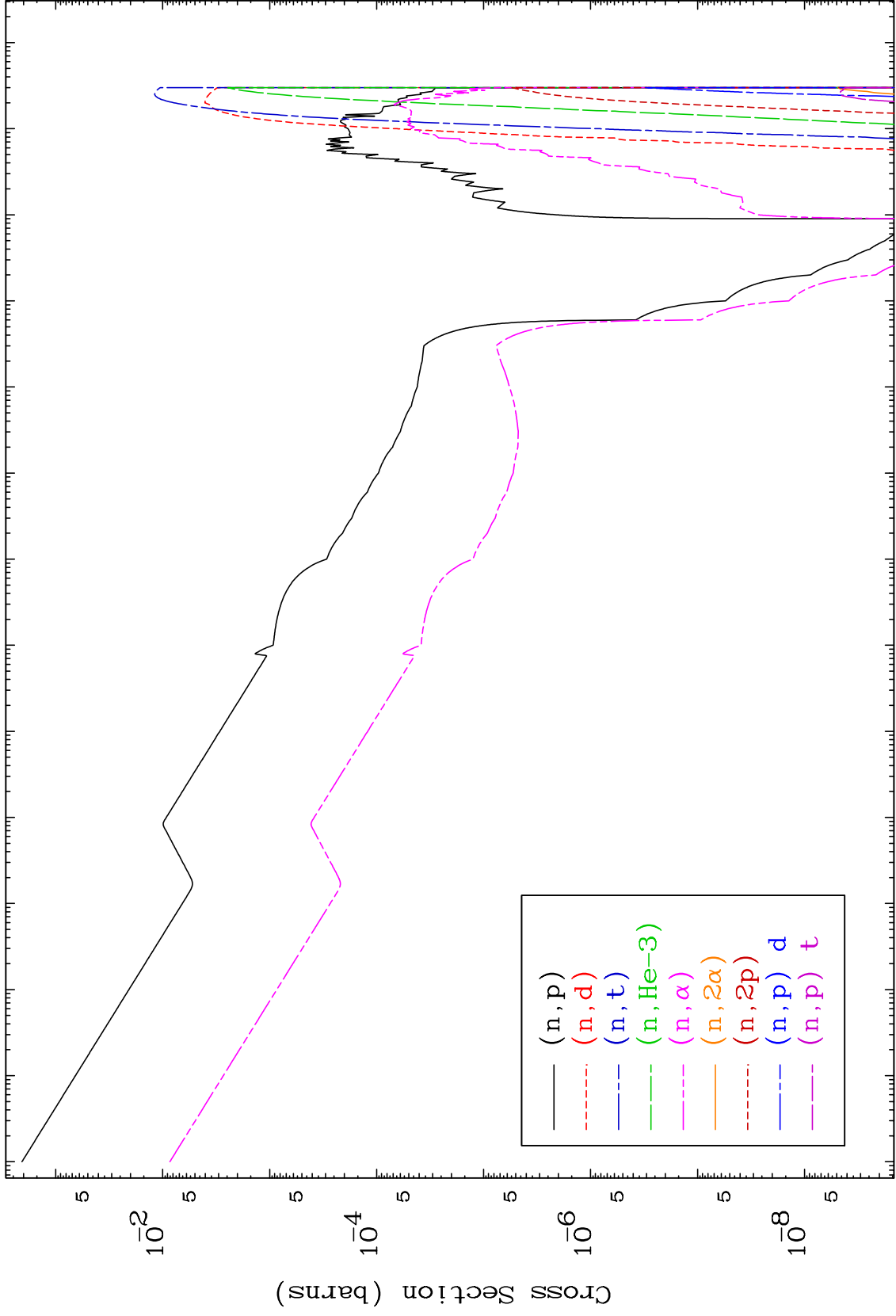
101-Md-246



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Charged Particle  
293 Kelvin Cross Sections

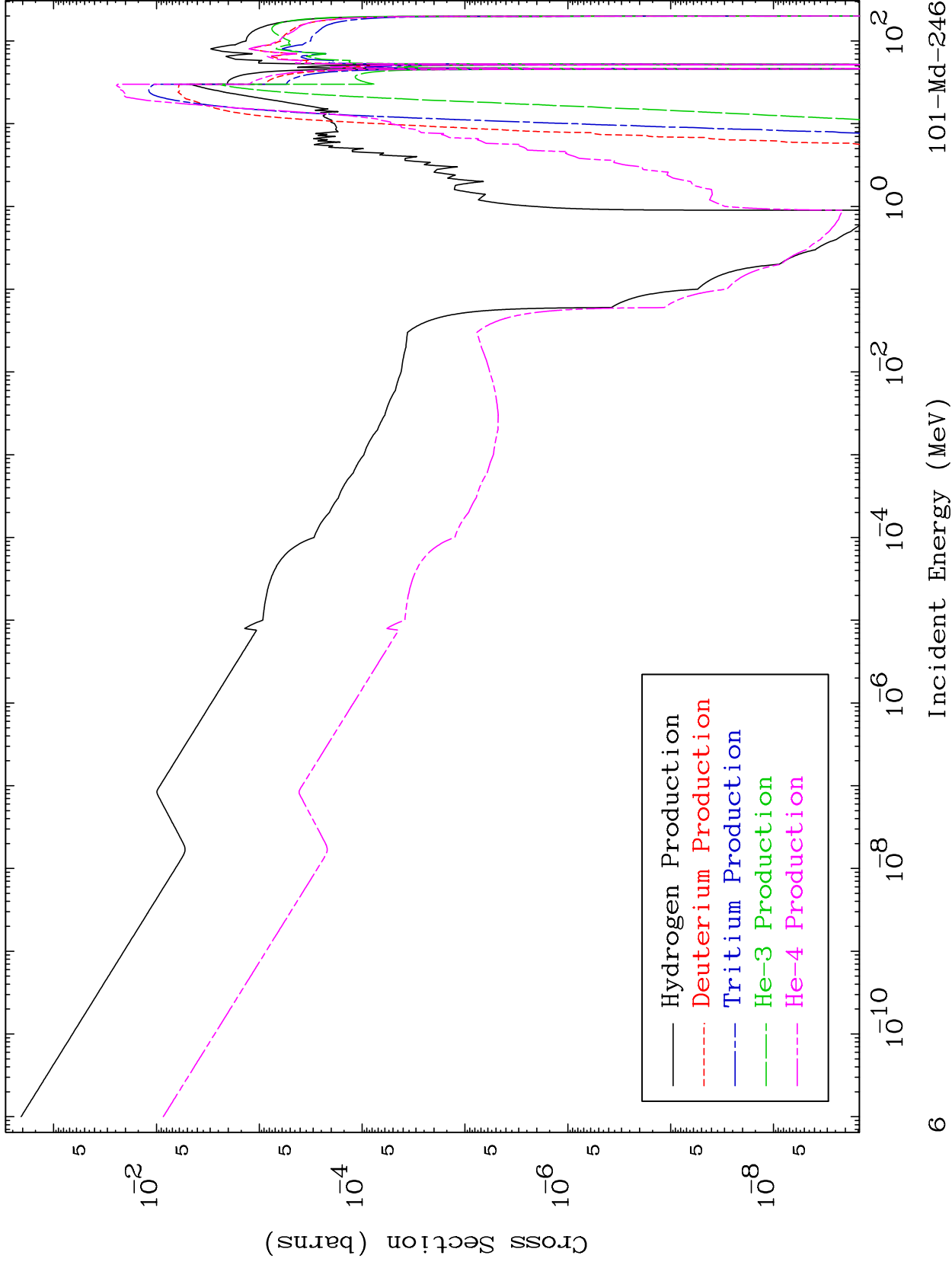
101-Md-246



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Particle Production  
293 Kelvin Cross Sections

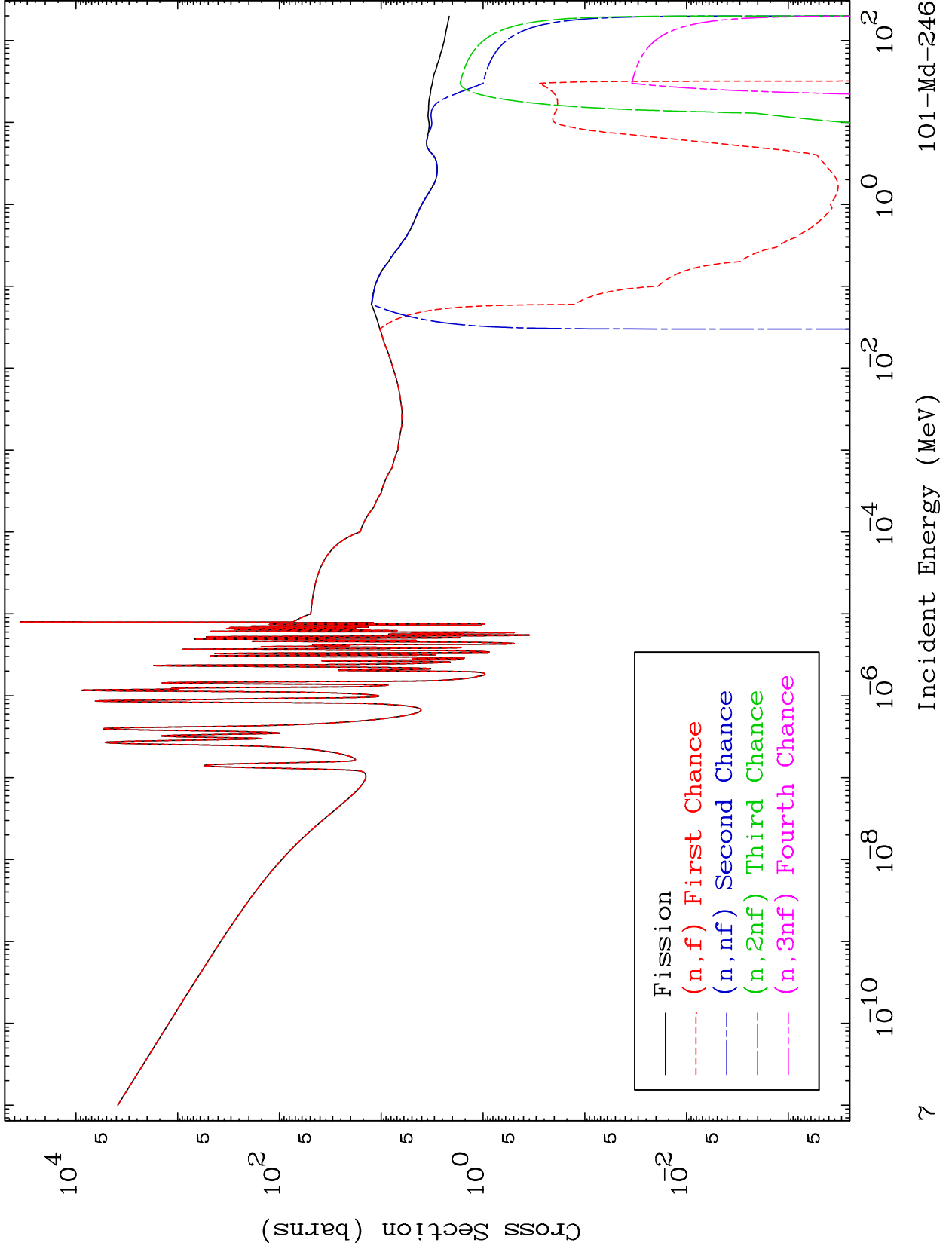
101-Md-246



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Fission  
293 Kelvin Cross Sections

101-Md-246

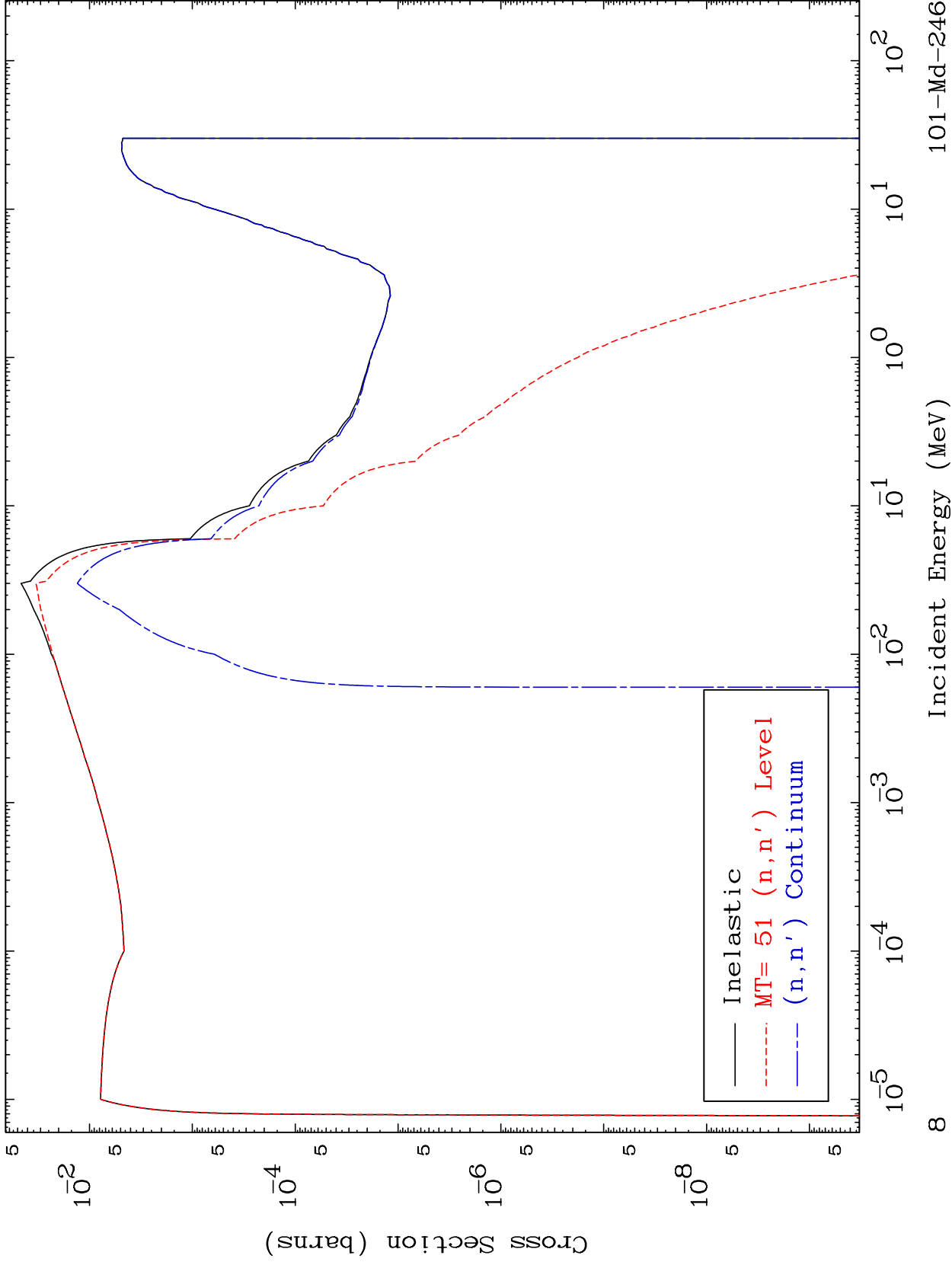




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(n,n') Level  
293 Kelvin Cross Sections

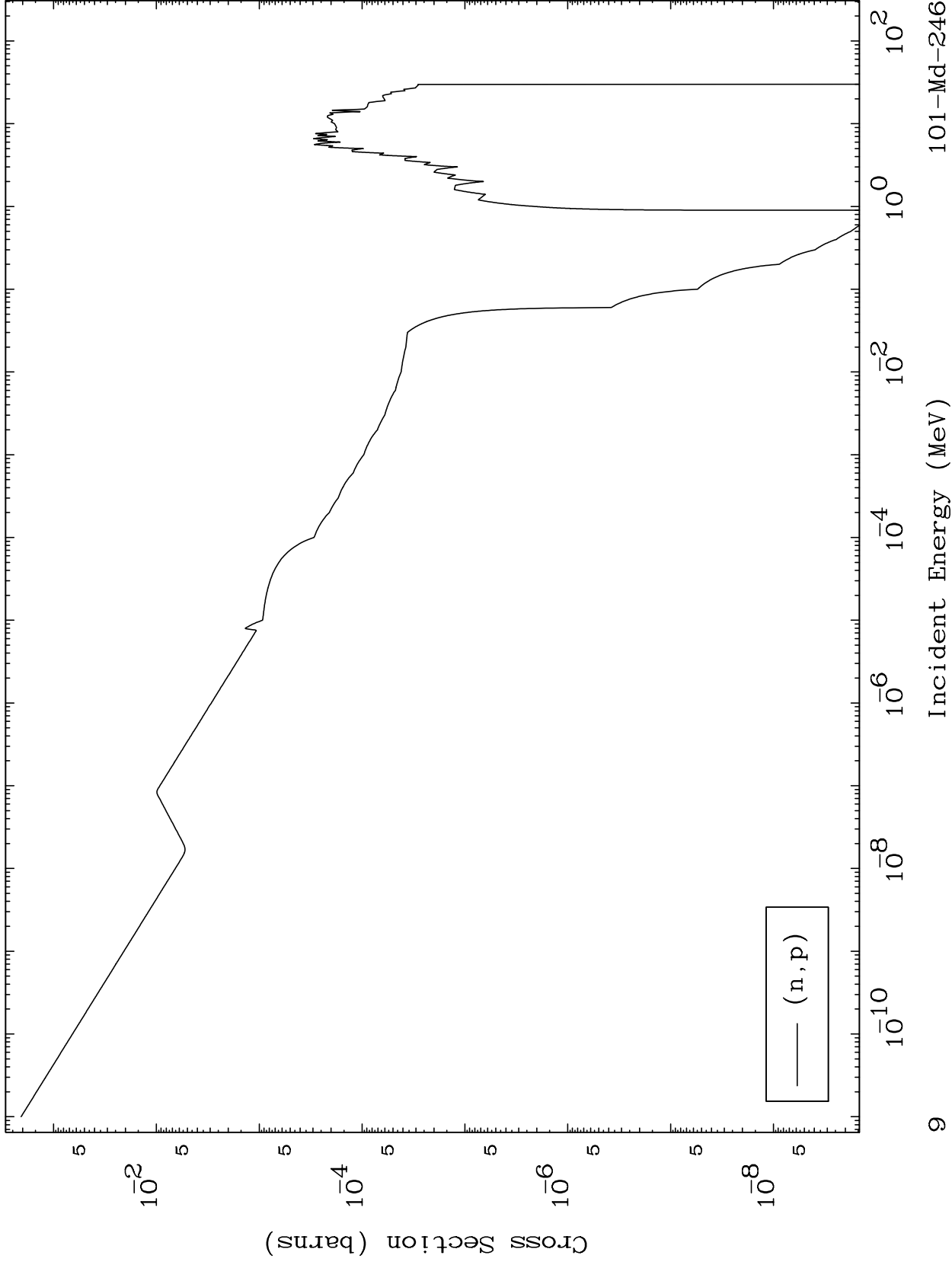
101-Md-246



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(n,p) Levels  
293 Kelvin Cross Sections

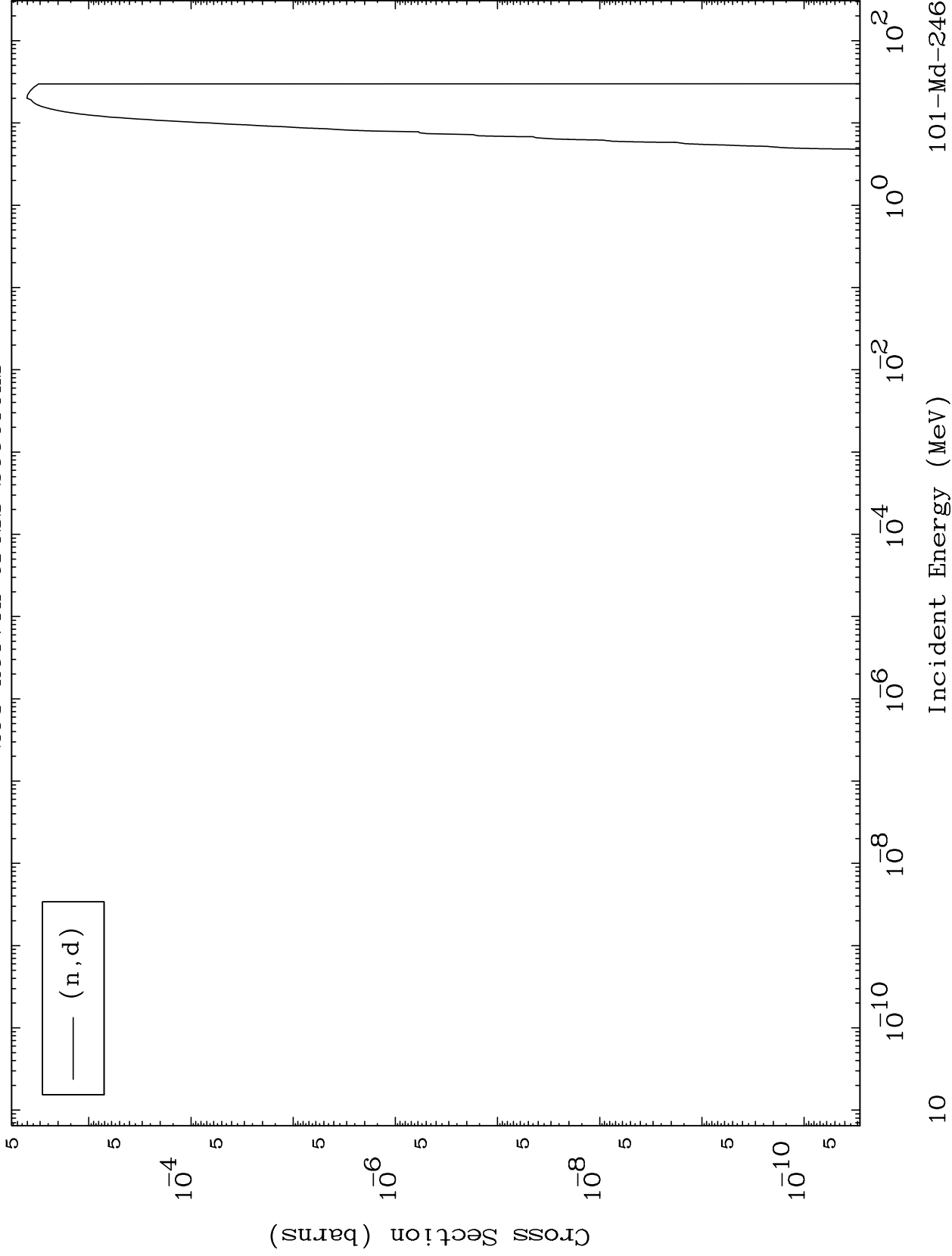
101-Md-246



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(n,d) Levels  
293 Kelvin Cross Sections

101-Md-246



10

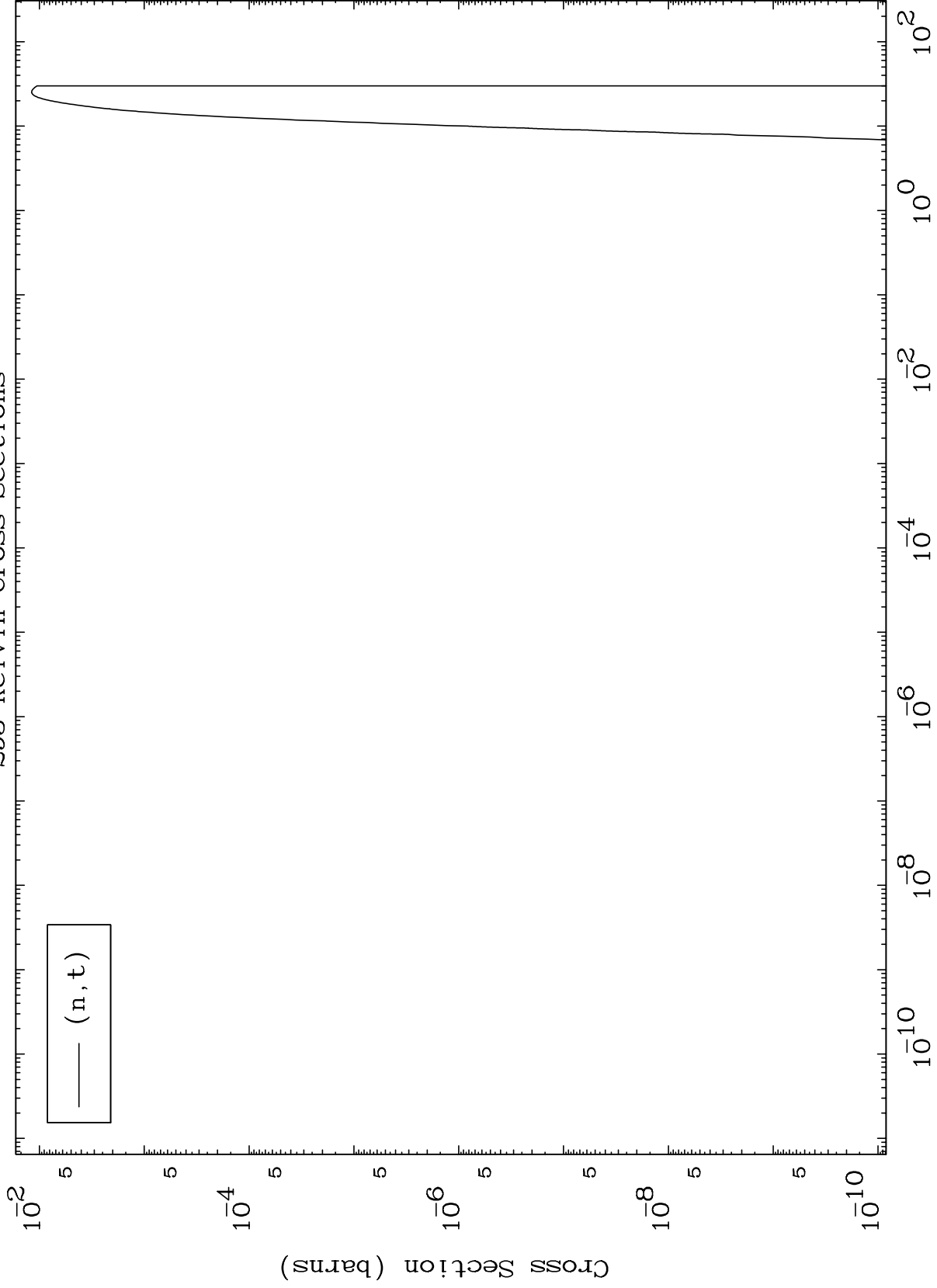
Incident Energy (MeV)

101-Md-246

MAT 9952

(n,t) Levels  
293 Kelvin Cross Sections

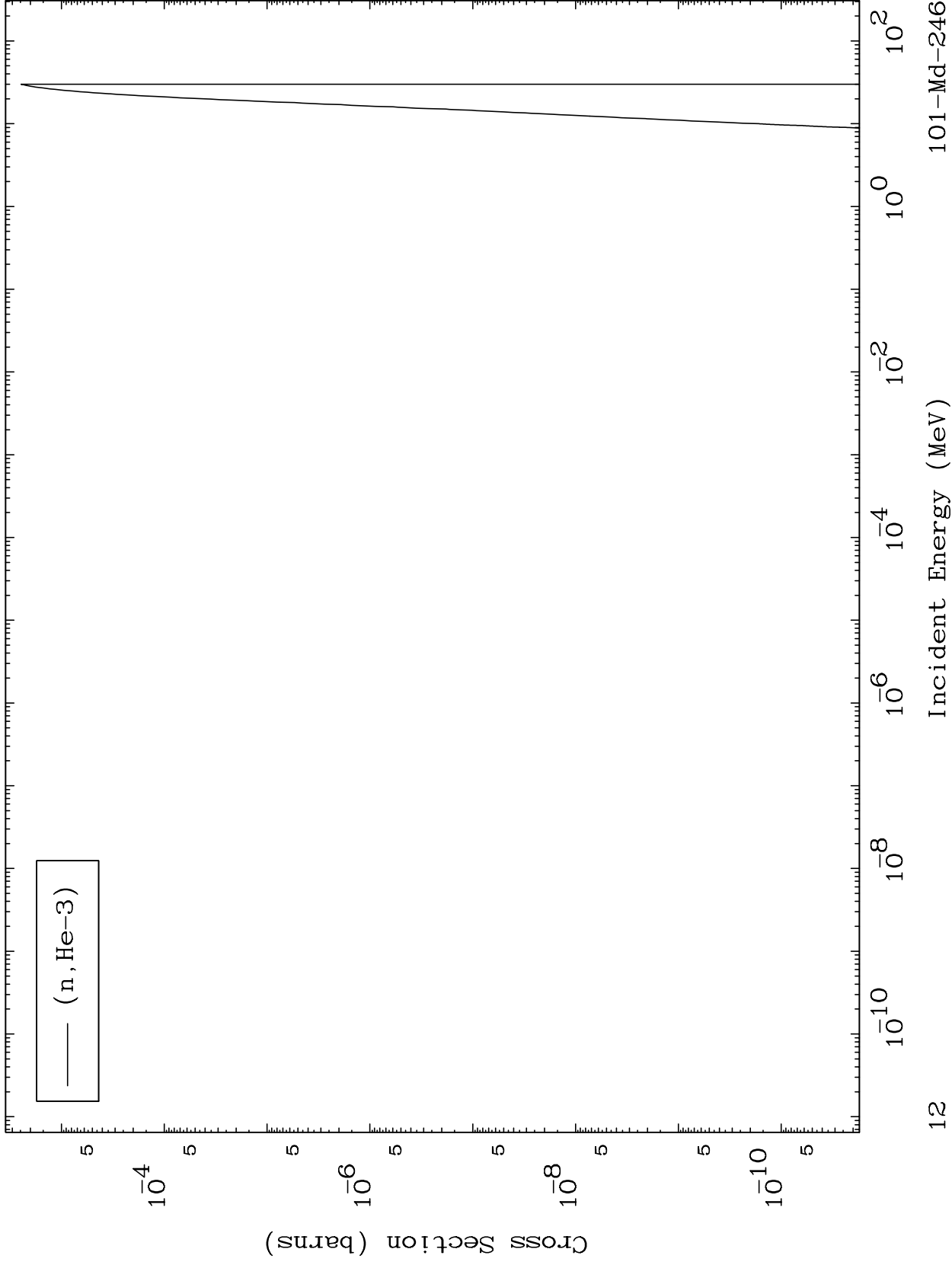
101-Md-246



MAT 9952

(n,He3) Levels  
293 Kelvin Cross Sections

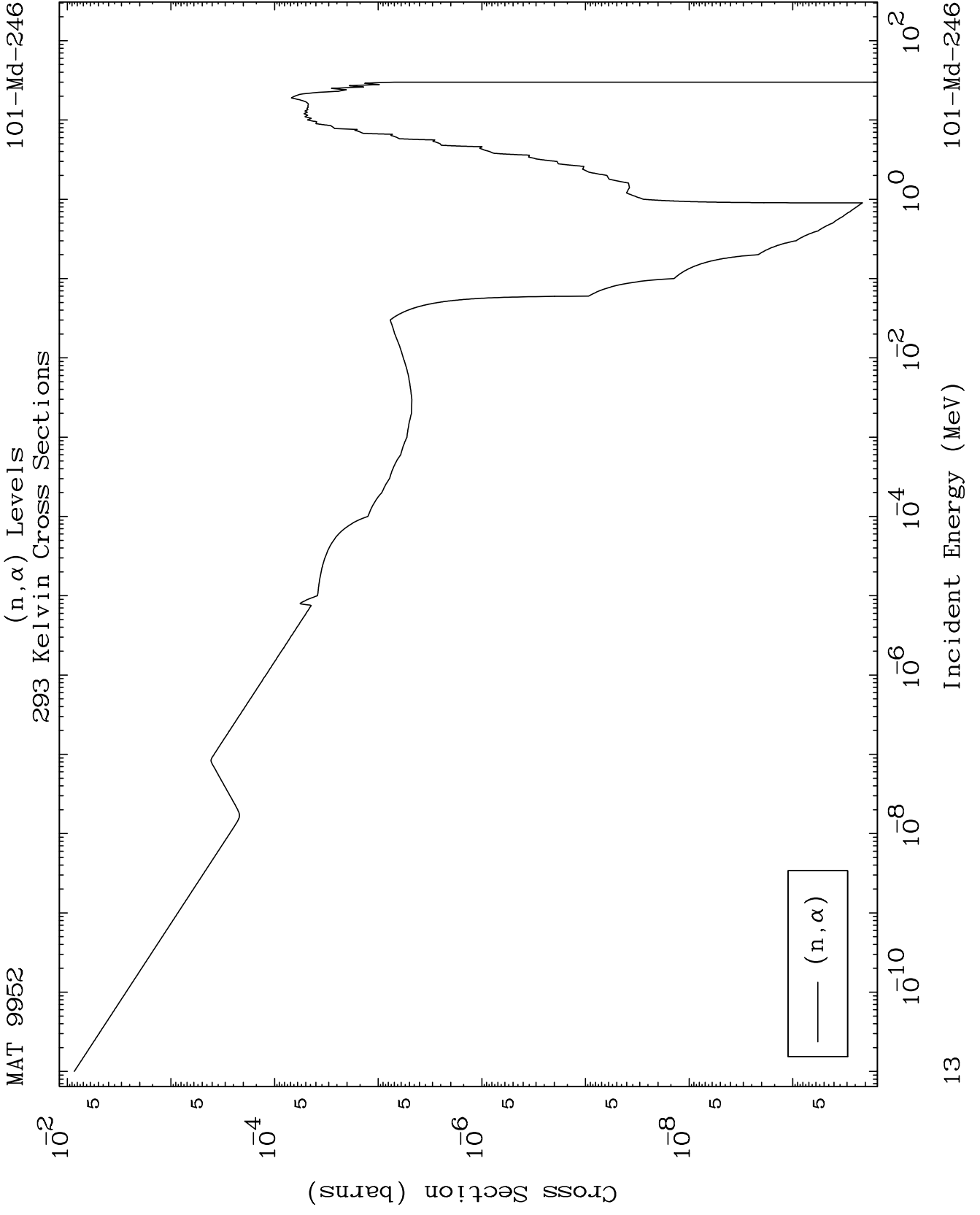
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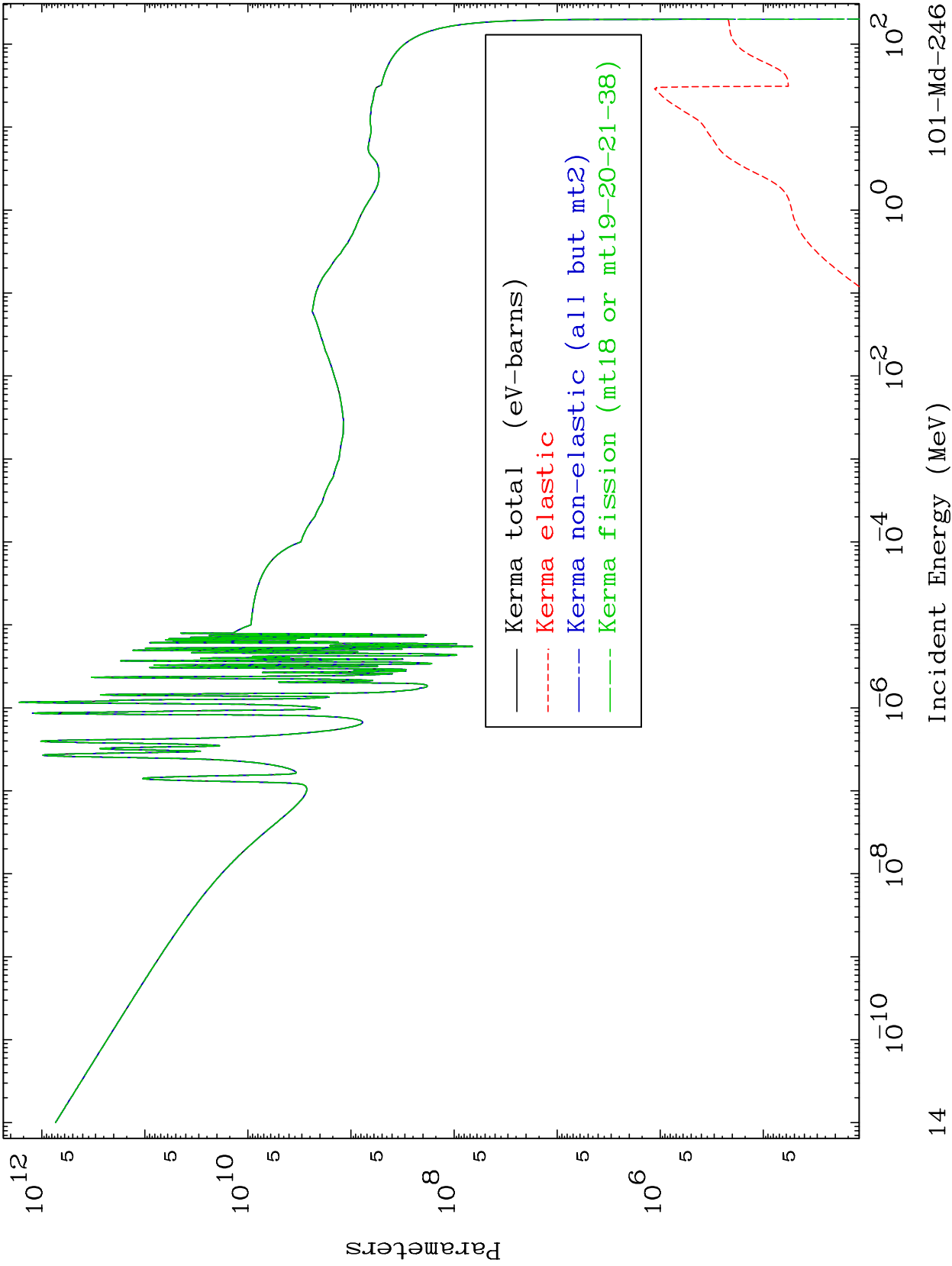


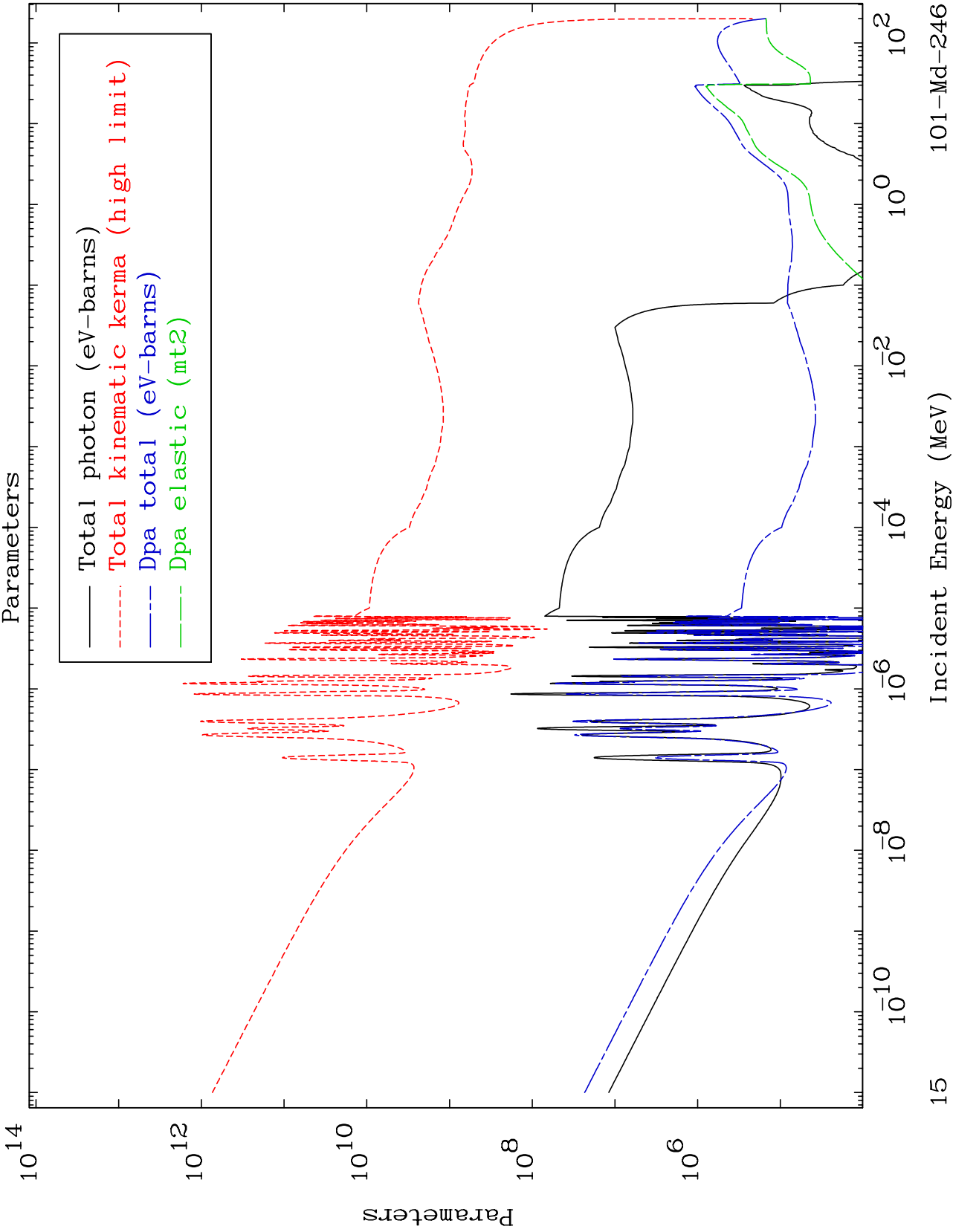
12

Incident Energy (MeV)

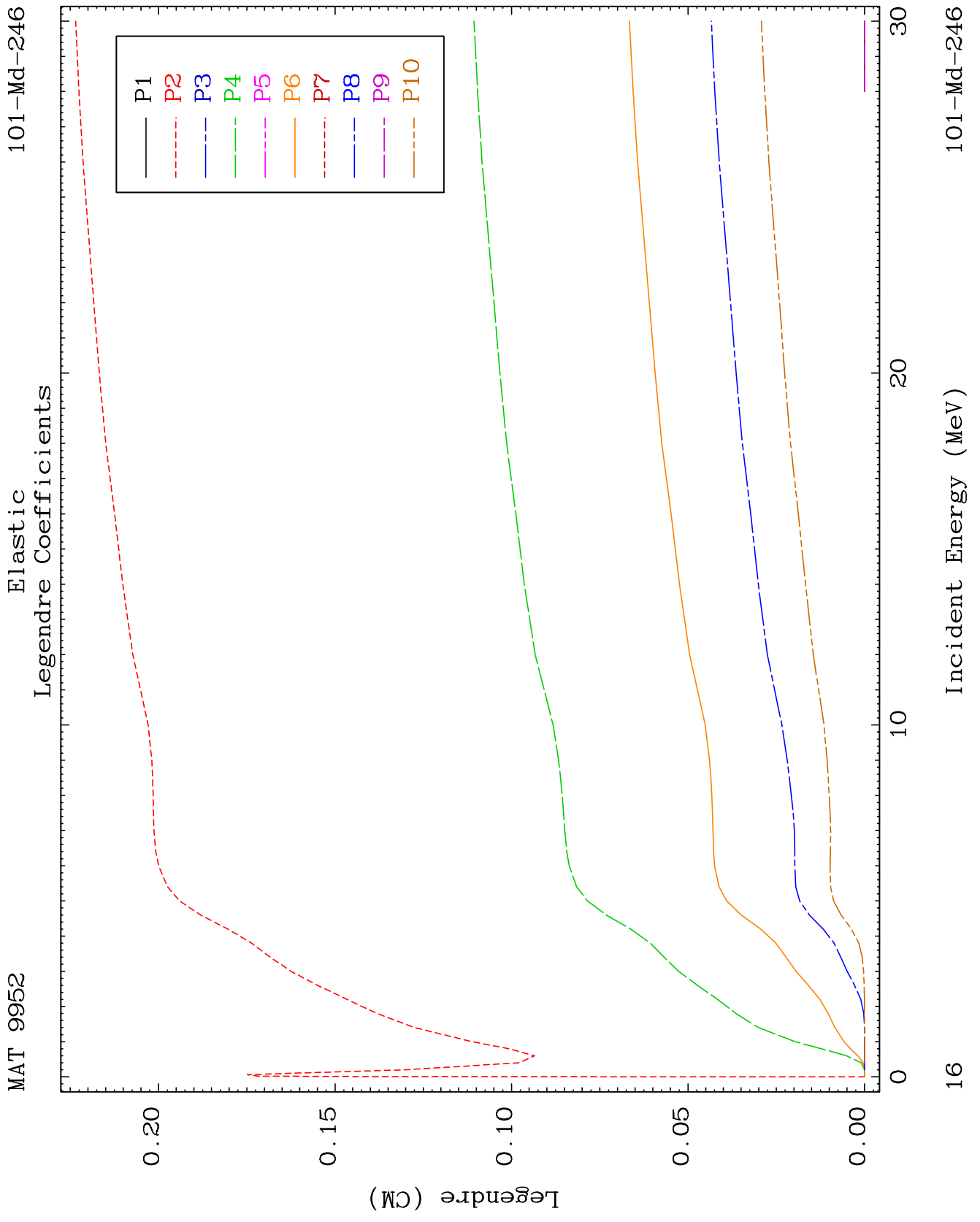
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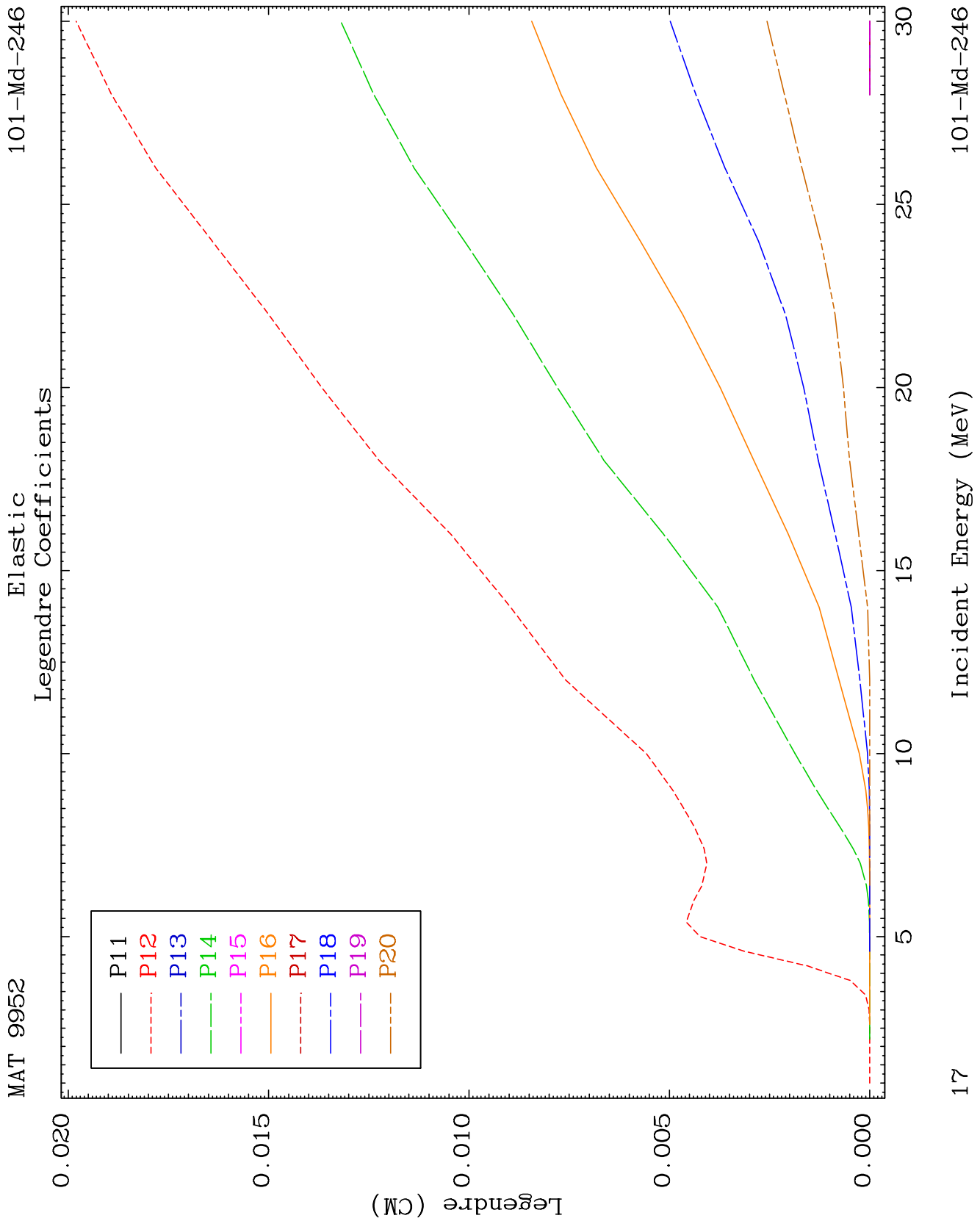








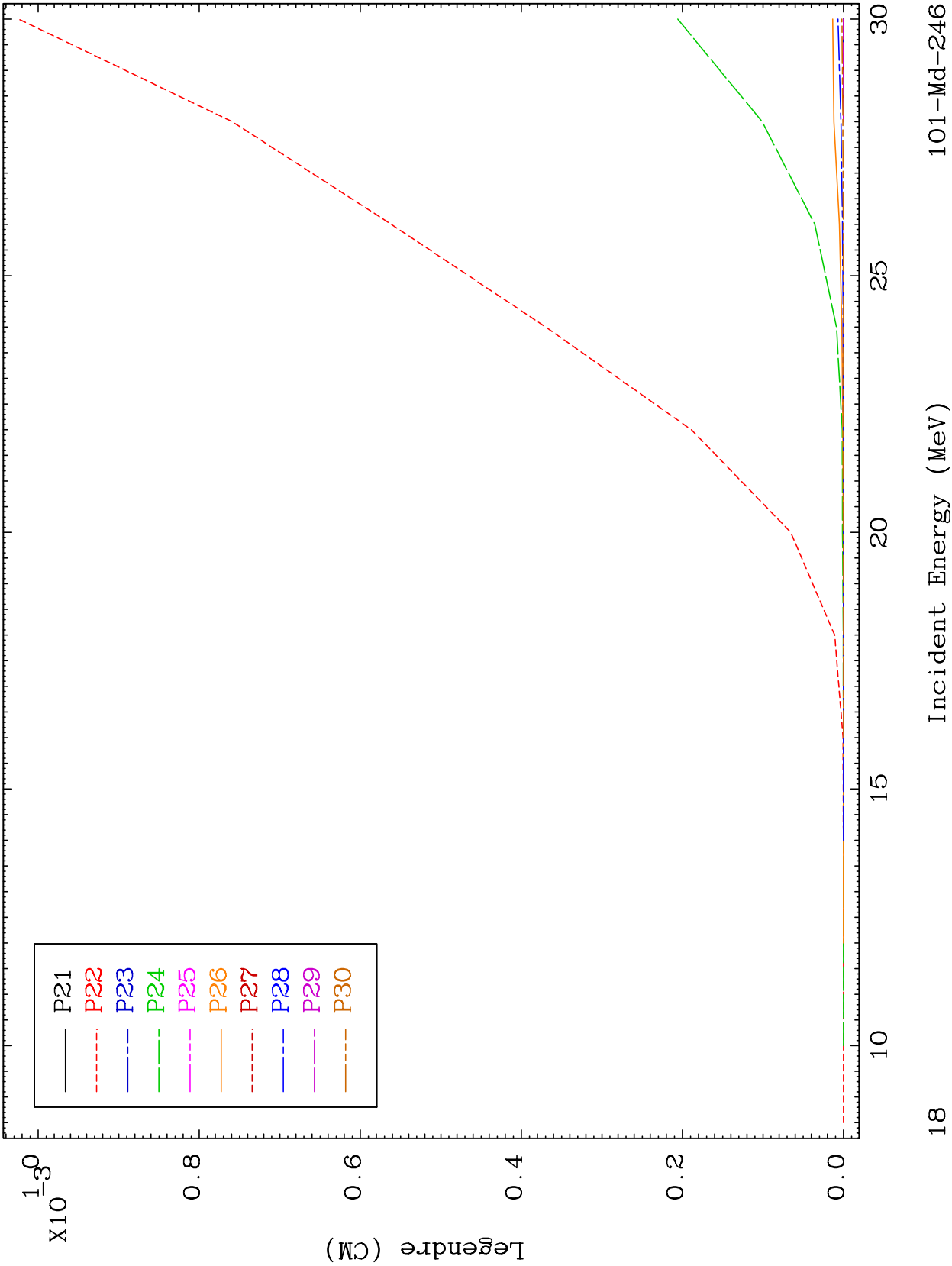




MAT 9952

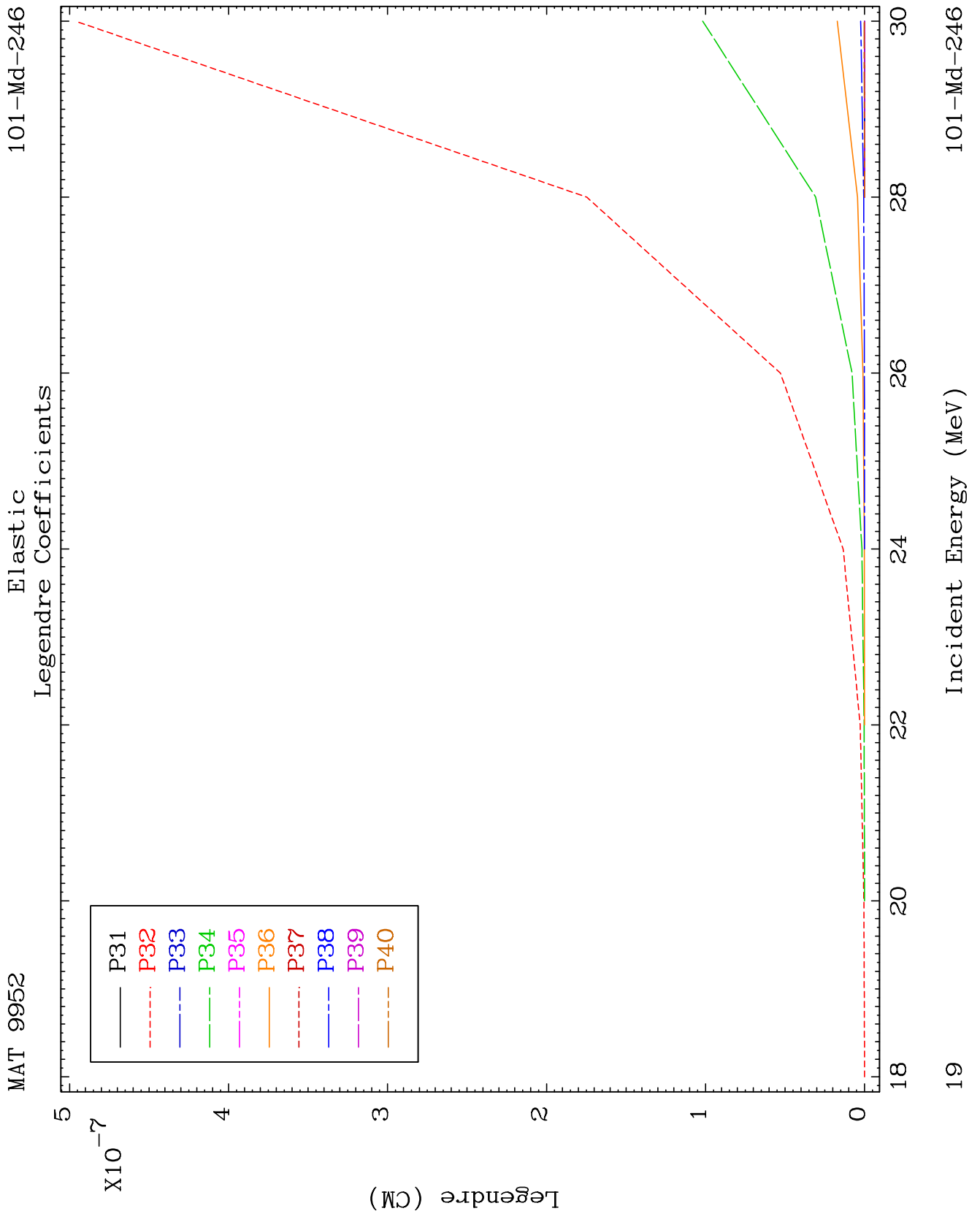
### Elastic Legendre Coefficients

101-Md-246



18

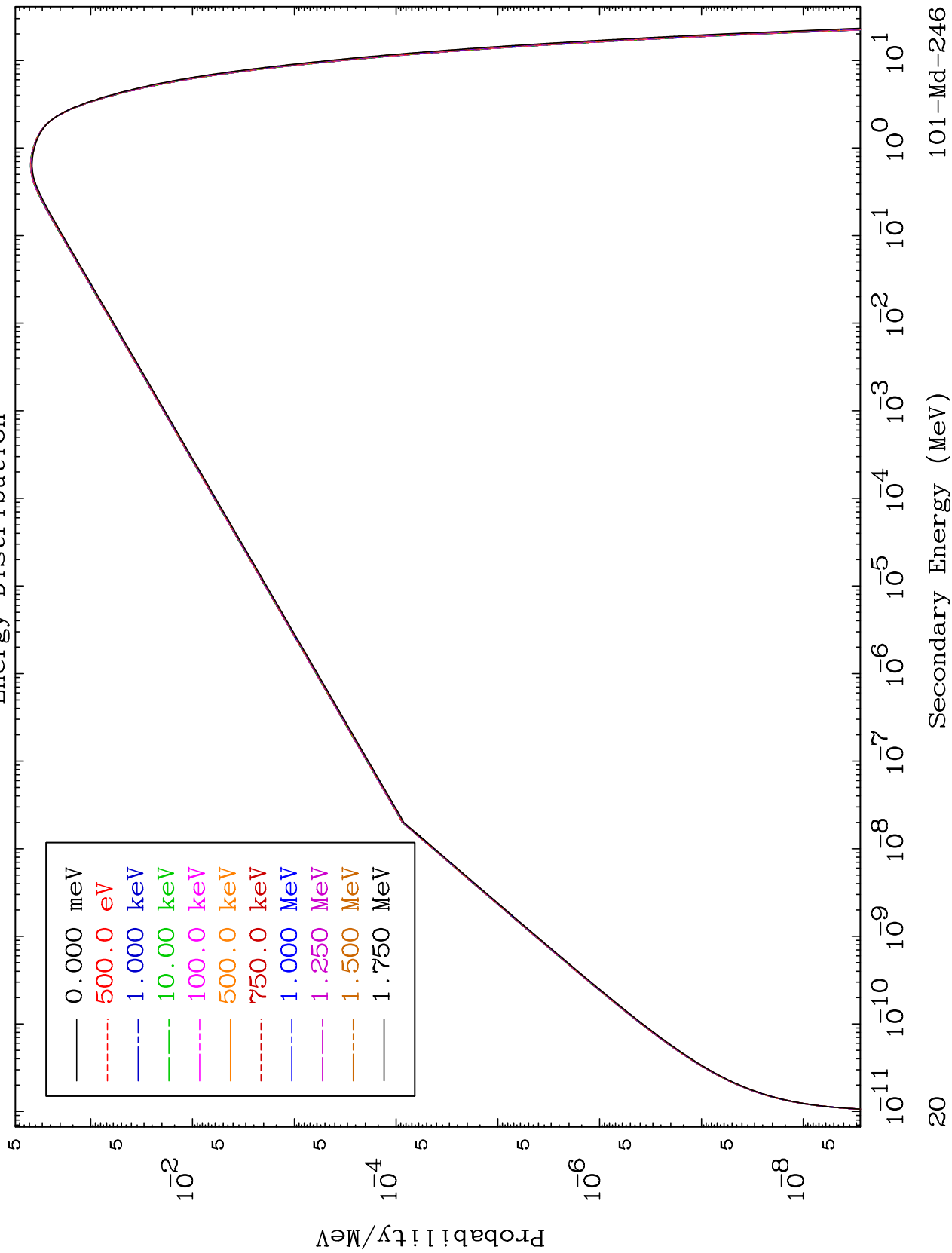
101-Md-246



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### Fission Energy Distribution

101-Md-246



101-Md-246

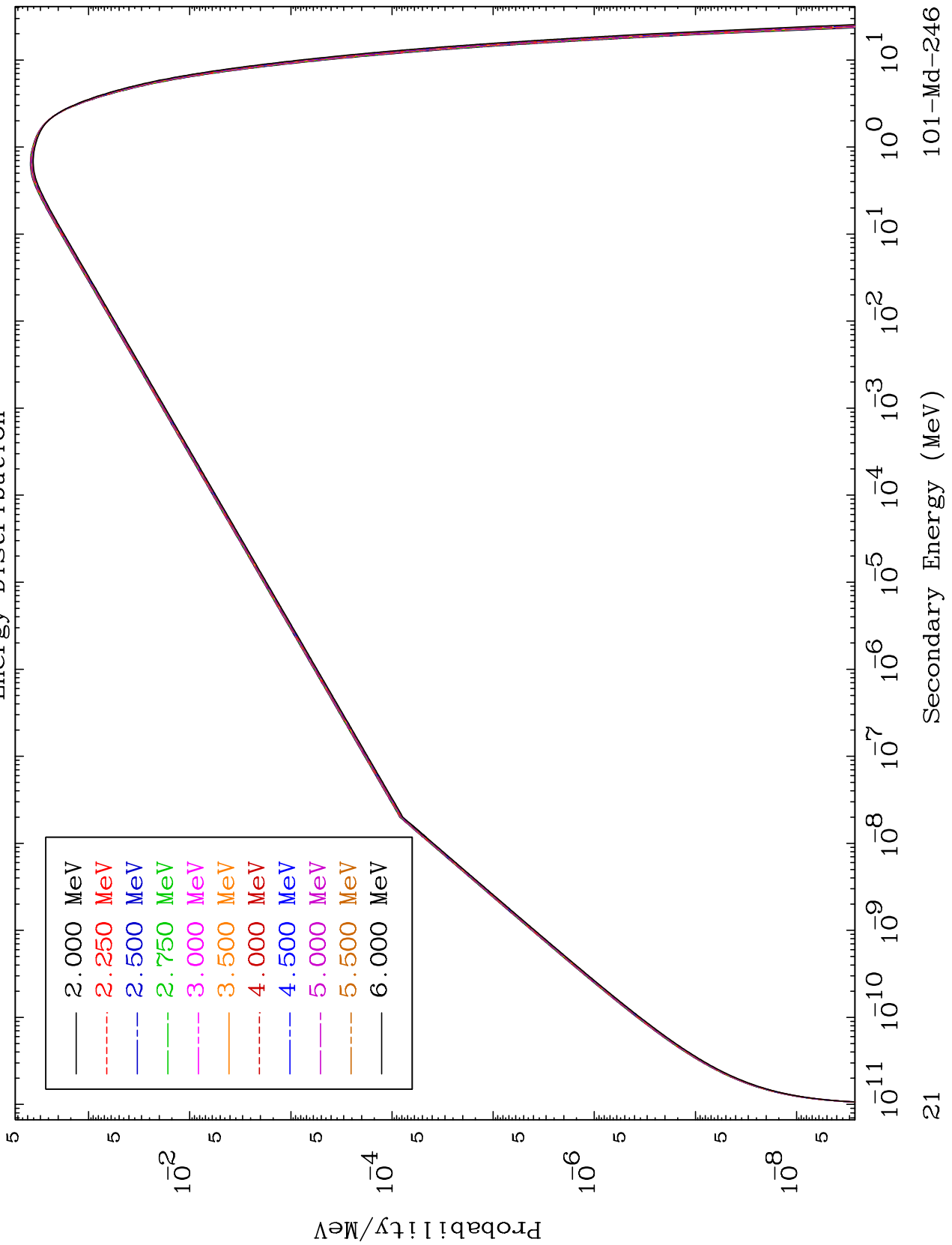
Secondary Energy (MeV)

20

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Fission  
Energy Distribution

101-Md-246

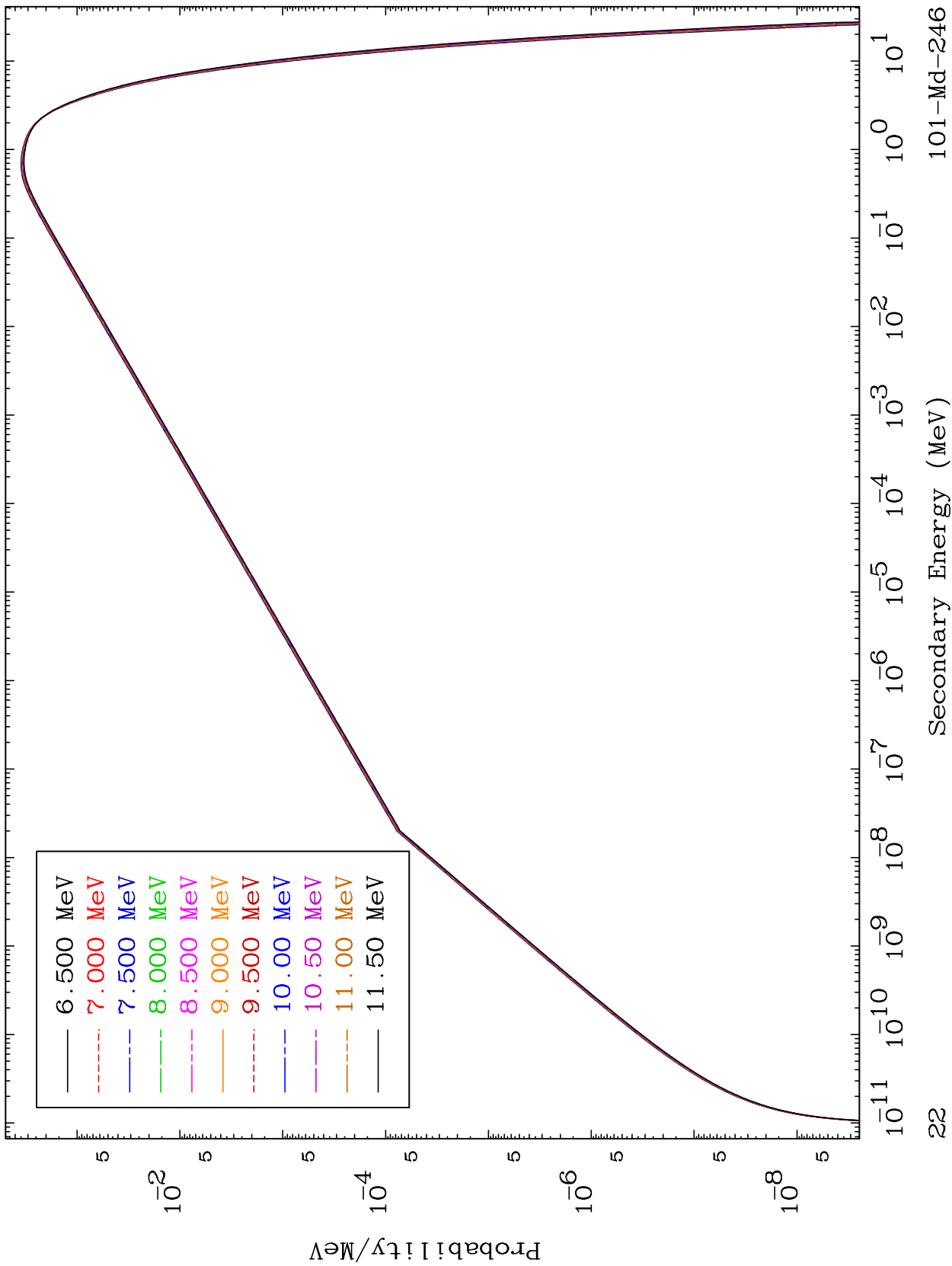


101-Md-246

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### Fission Energy Distribution

101-Md-246



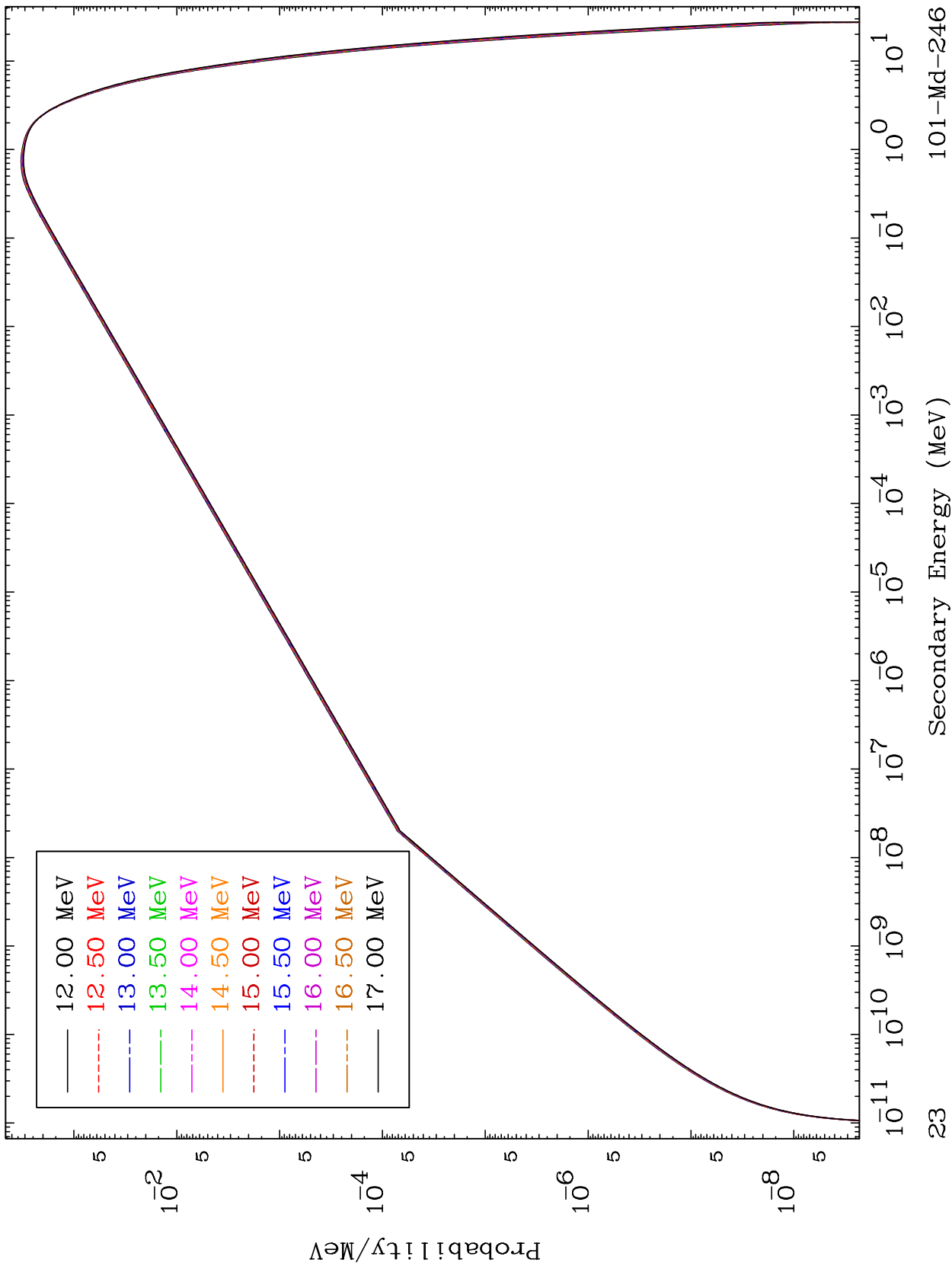
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Secondary Energy (MeV)

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### Fission Energy Distribution

101-Md-246



101-Md-246

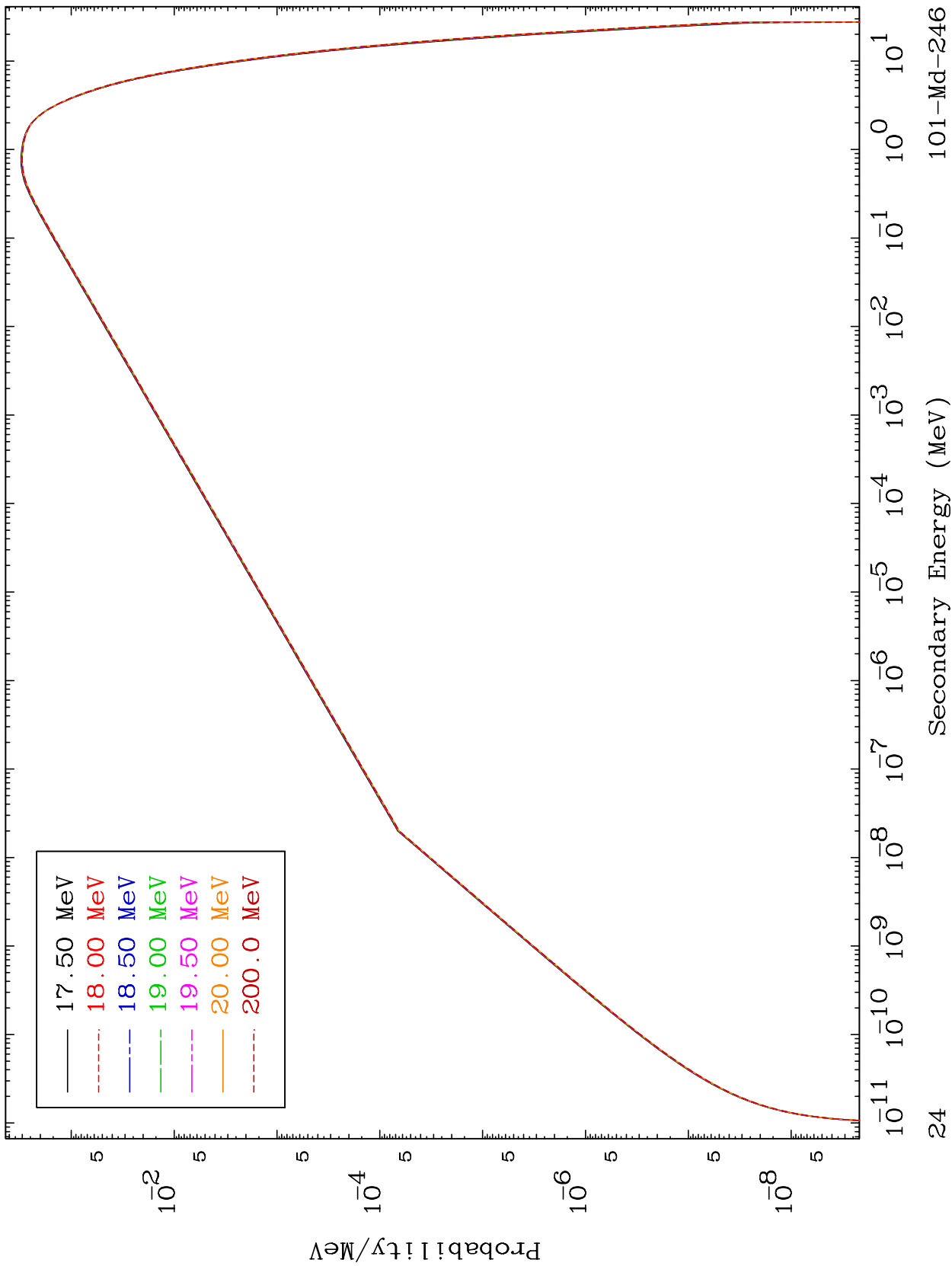
Secondary Energy (MeV)



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### Fission Energy Distribution

101-Md-246

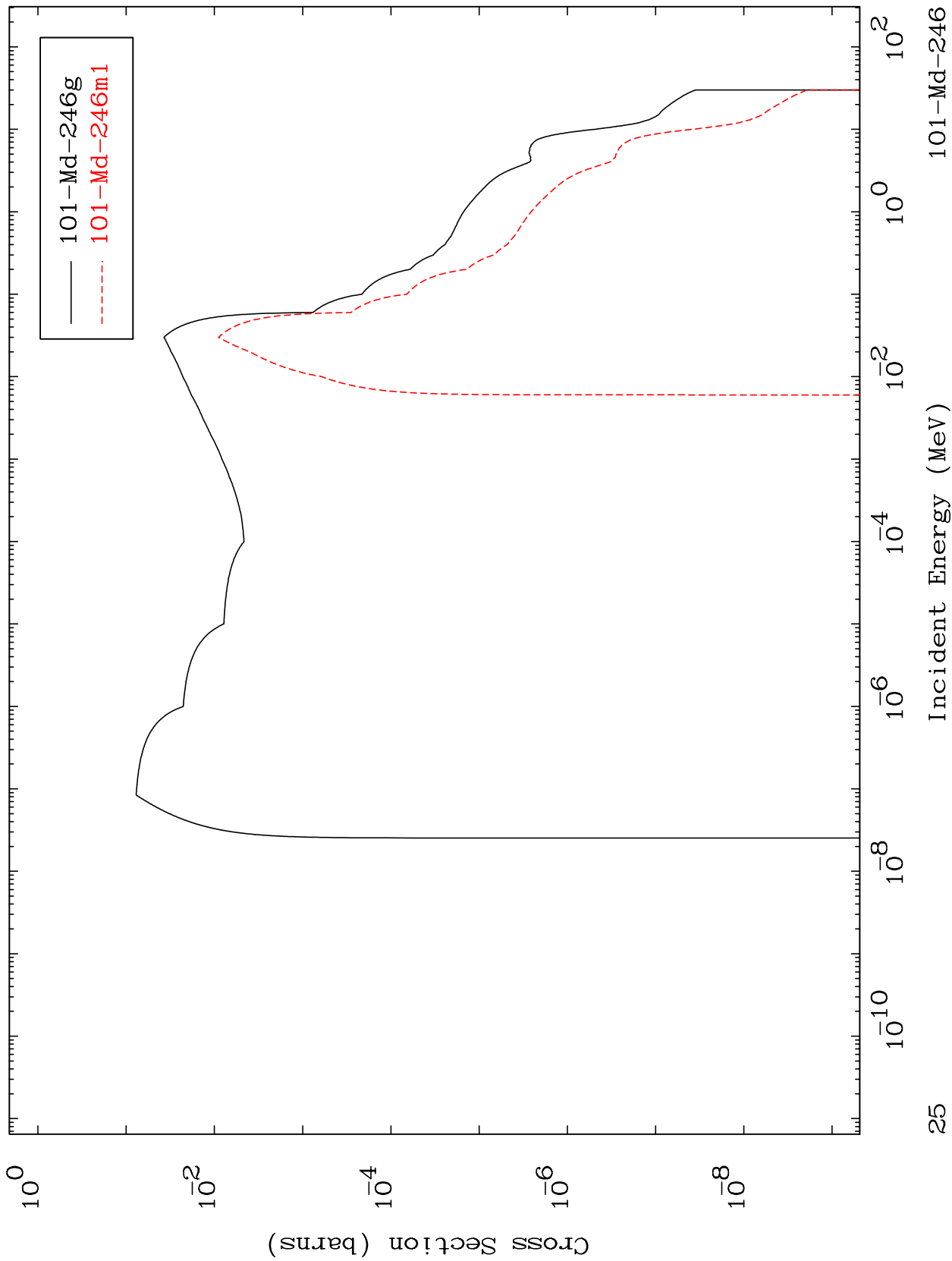


101-Md-246

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101-Md-246

Inelastic  
Radionuclide Production Cross Section



25

101-Md-246