

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

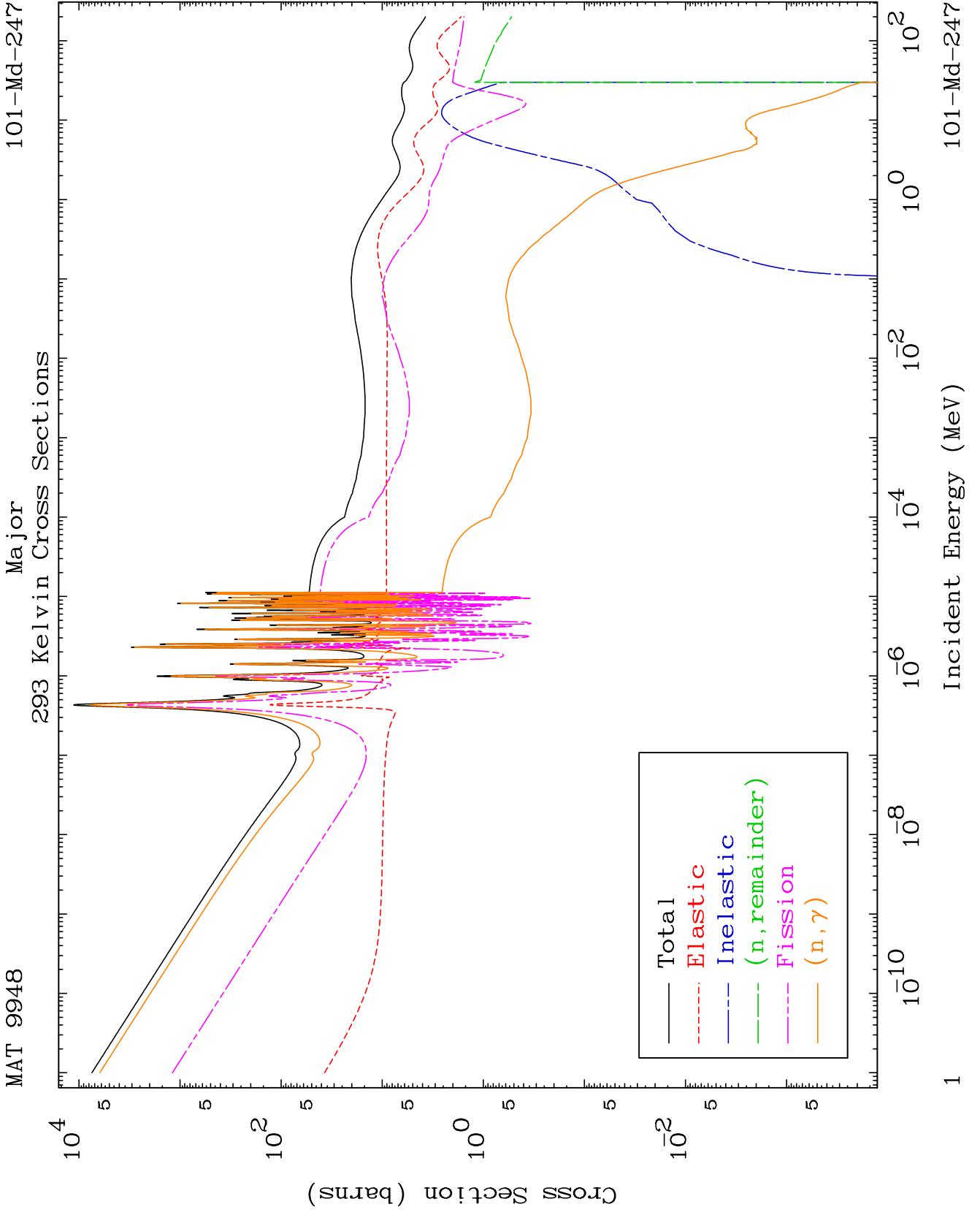
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E.Mail: redcullen1@comcast.net

Web: redcullen1.net/HOMEPAGE.NEW

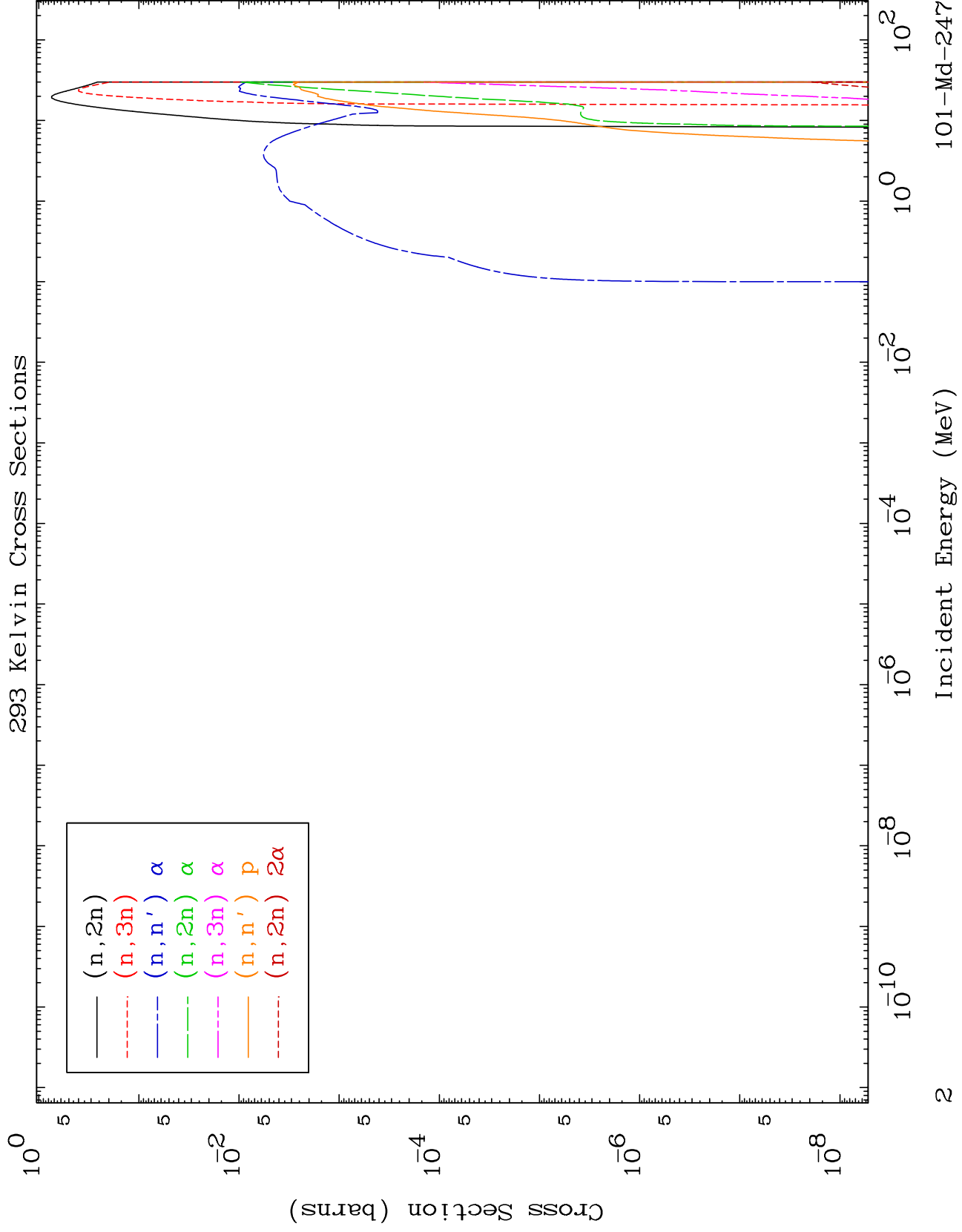
Press Mouse Button to Start



MAT 9948

Neutron Production
293 Kelvin Cross Sections

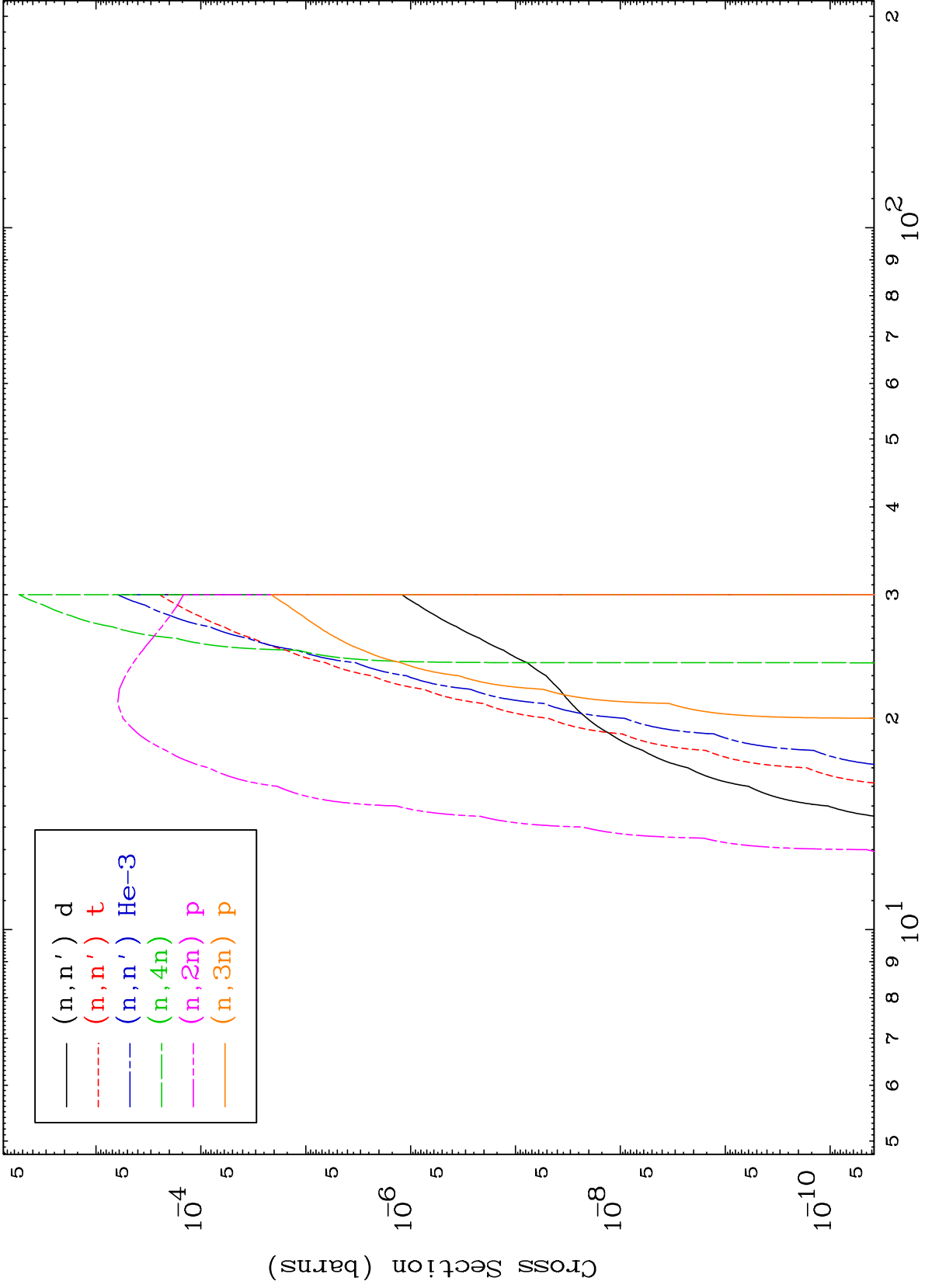
101-Md-247



MAT 9948

Neutron Production
293 Kelvin Cross Sections

101-Md-247



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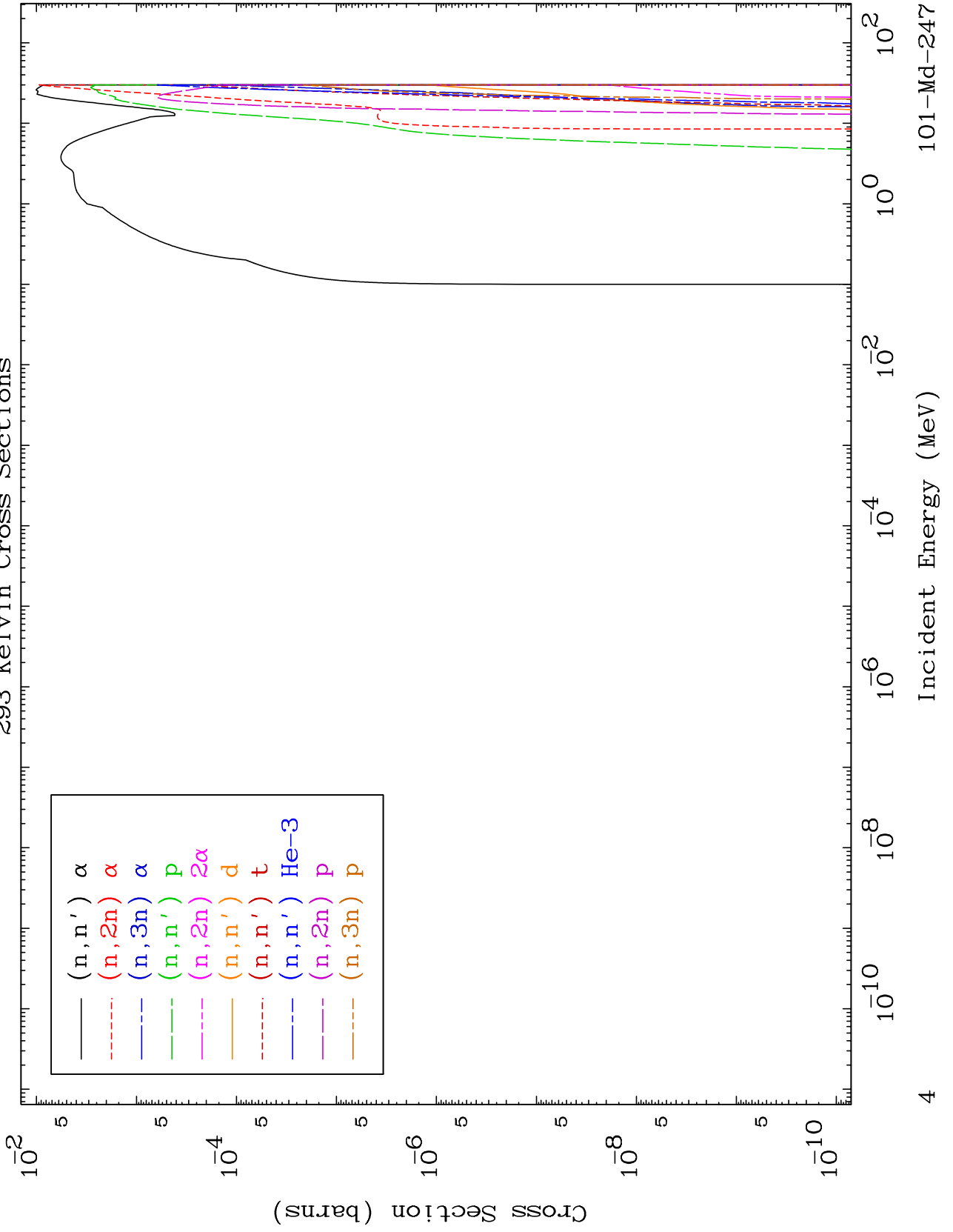
Incident Energy (MeV)

101-Md-247

MAT 9948

Charged Particle
293 Kelvin Cross Sections

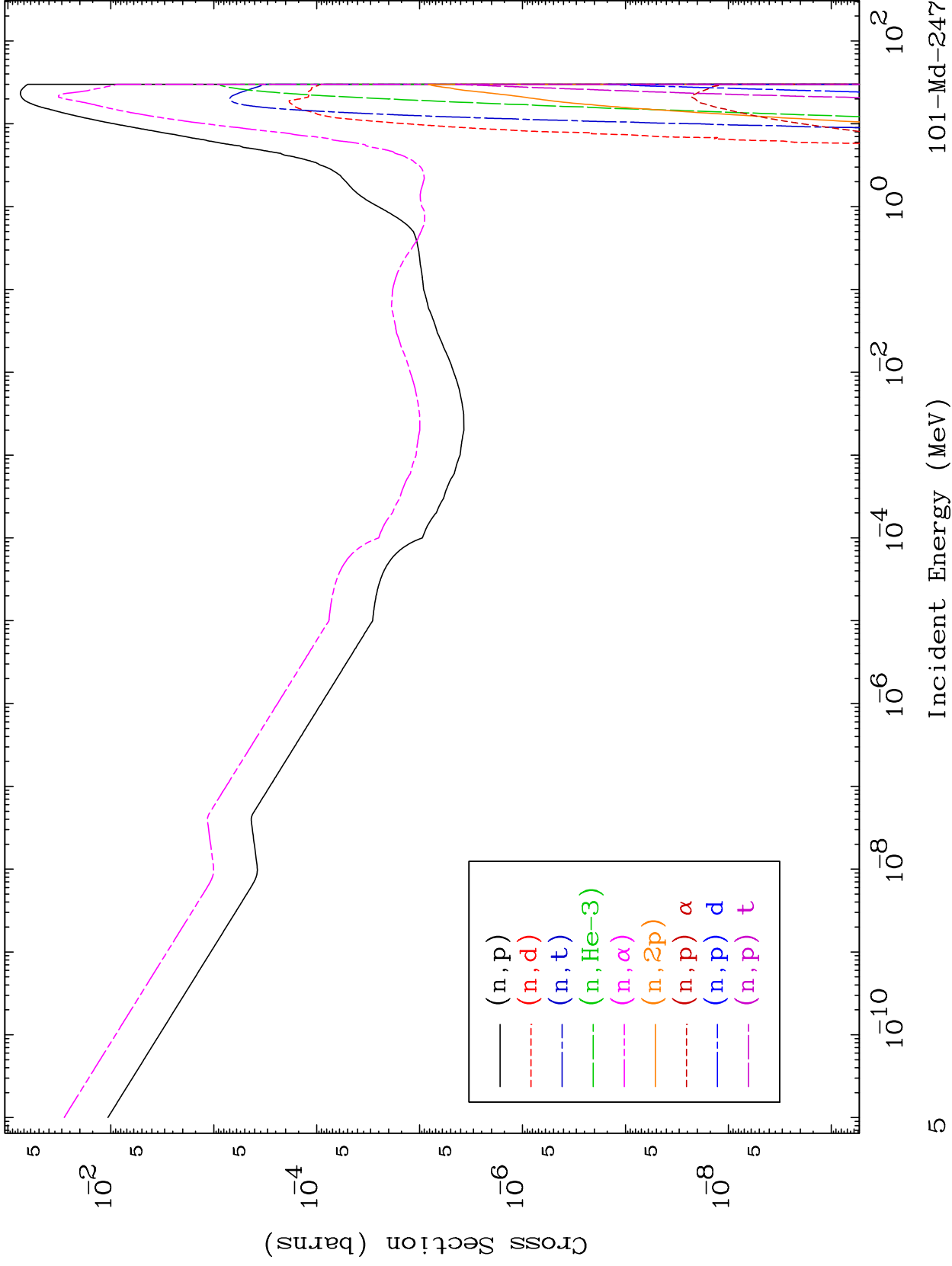
101-Md-247



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Charged Particle
293 Kelvin Cross Sections

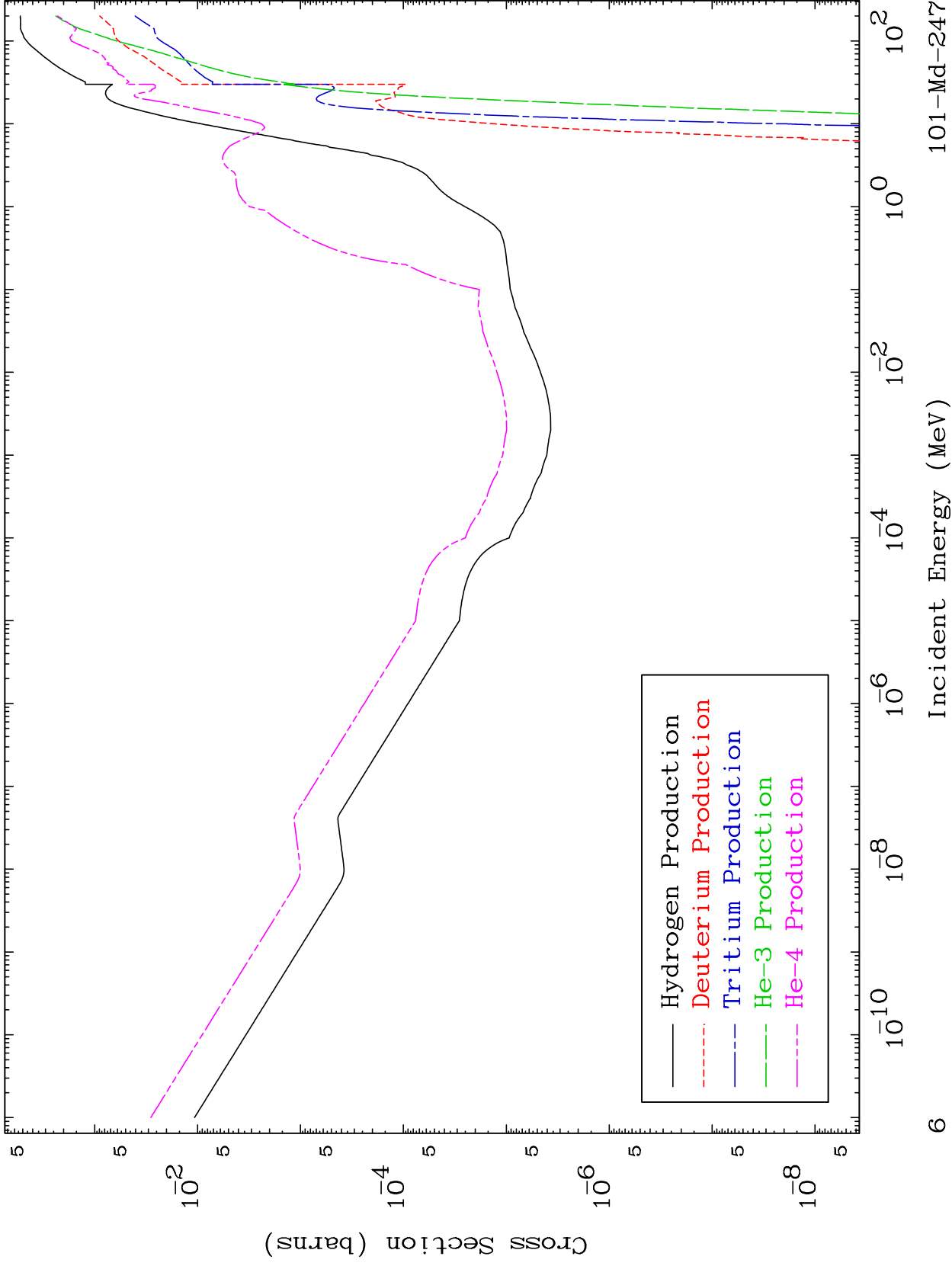
101-Md-247



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Particle Production
293 Kelvin Cross Sections

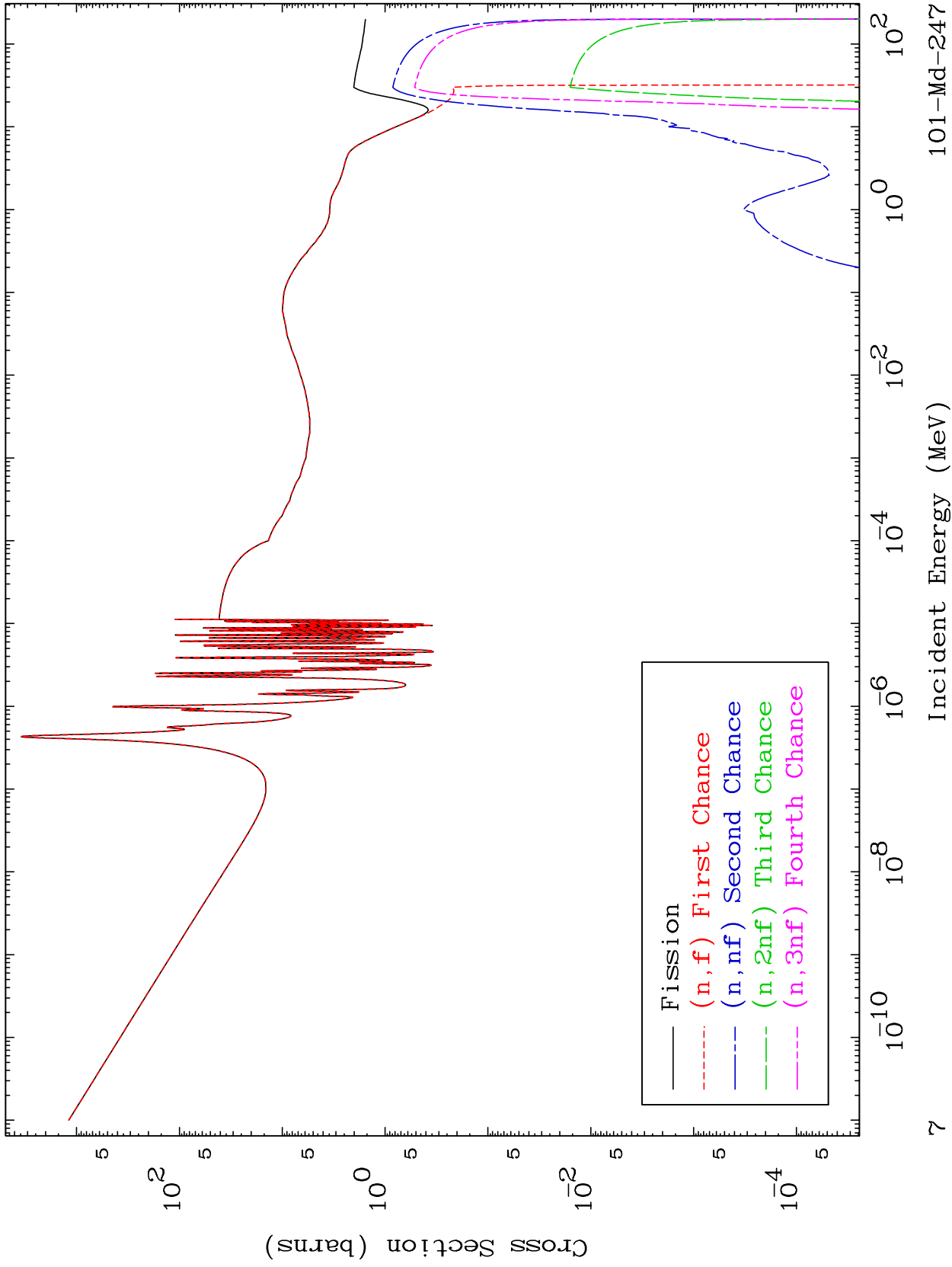
101-Md-247



MAT 9948

Fission
293 Kelvin Cross Sections

101-Md-247

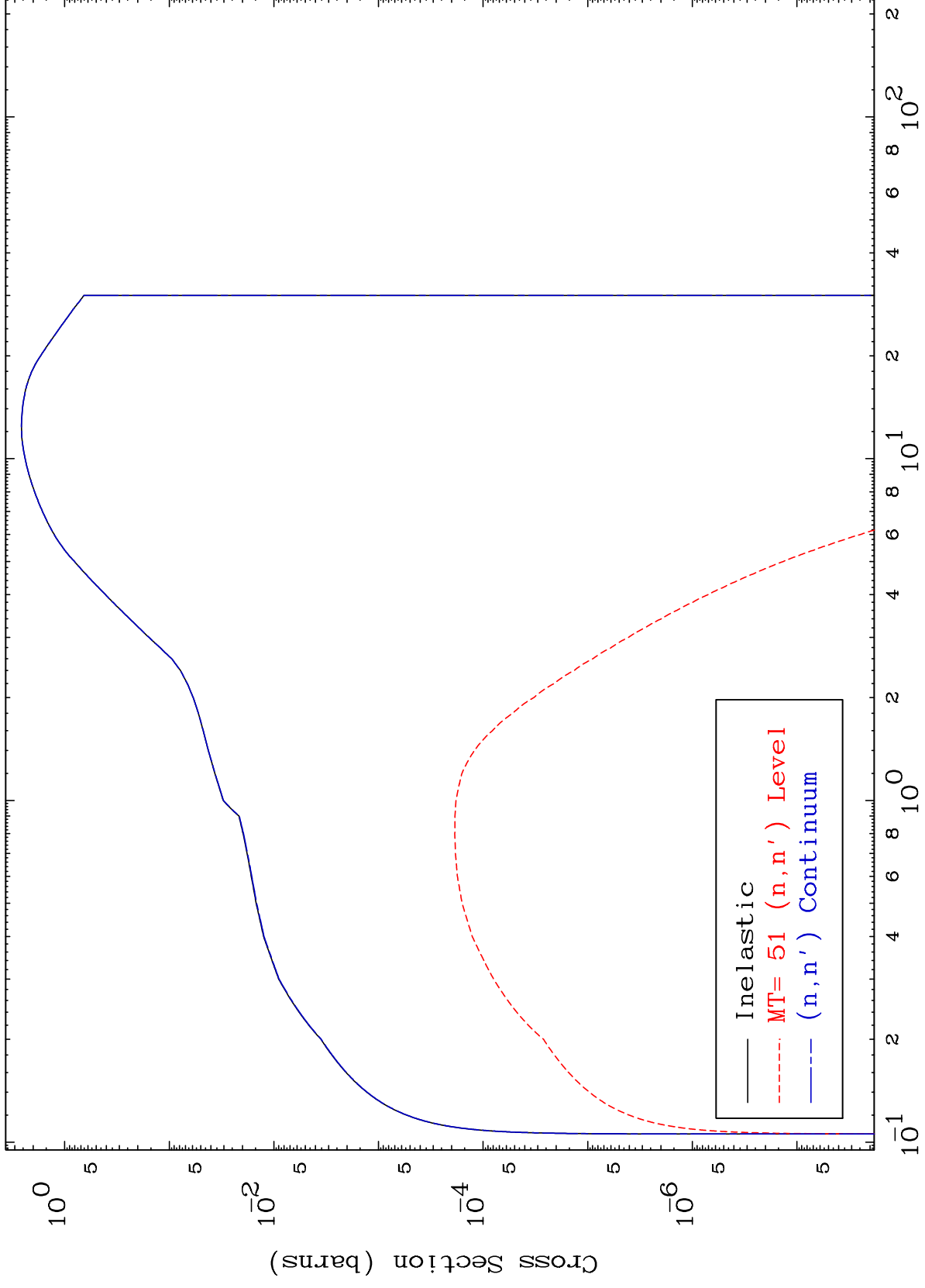


MAT 9948

(n,n') Level

101-Md-247

293 Kelvin Cross Sections



Incident Energy (MeV)

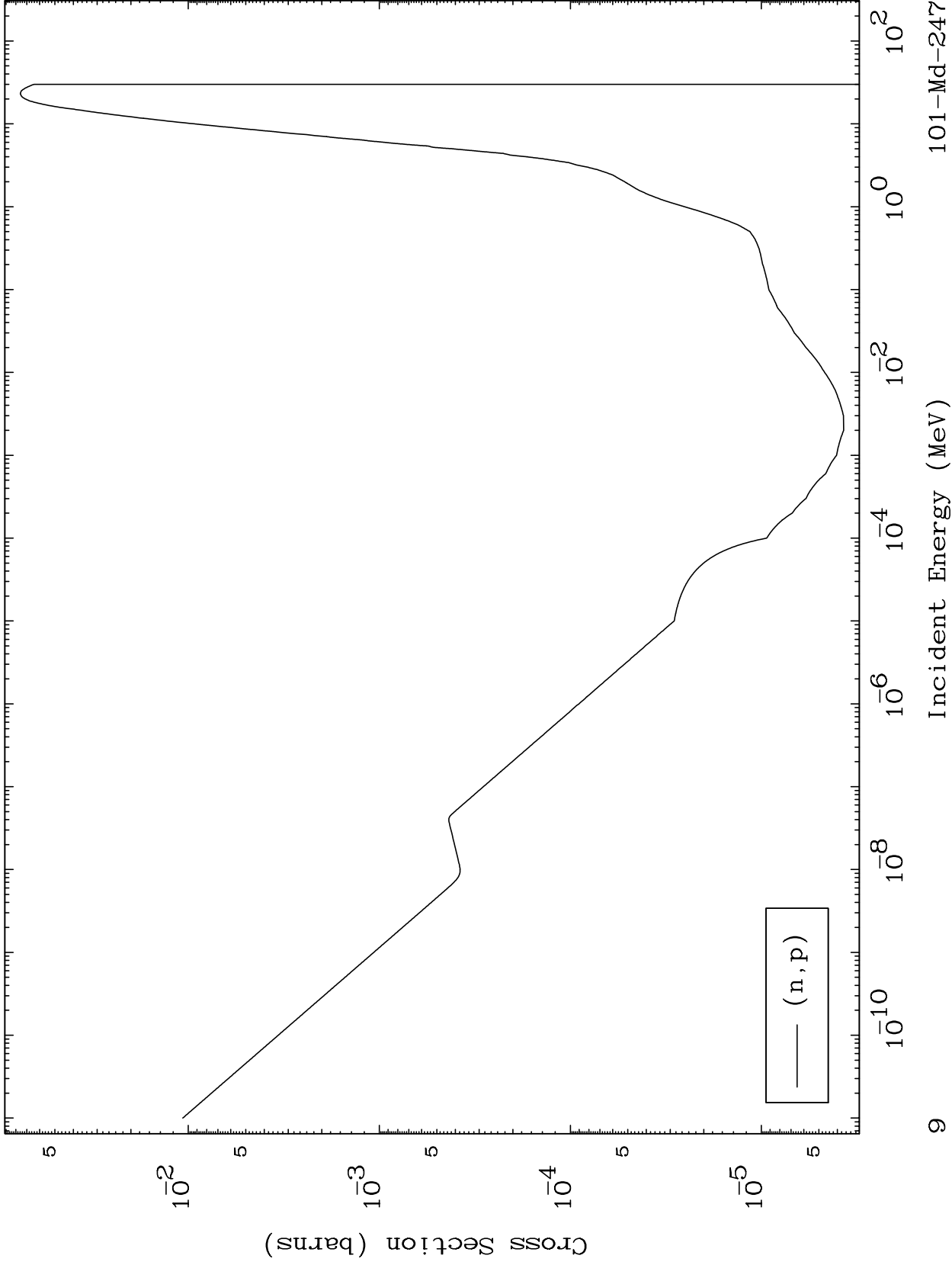
101-Md-247

8

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(n,p) Levels
293 Kelvin Cross Sections

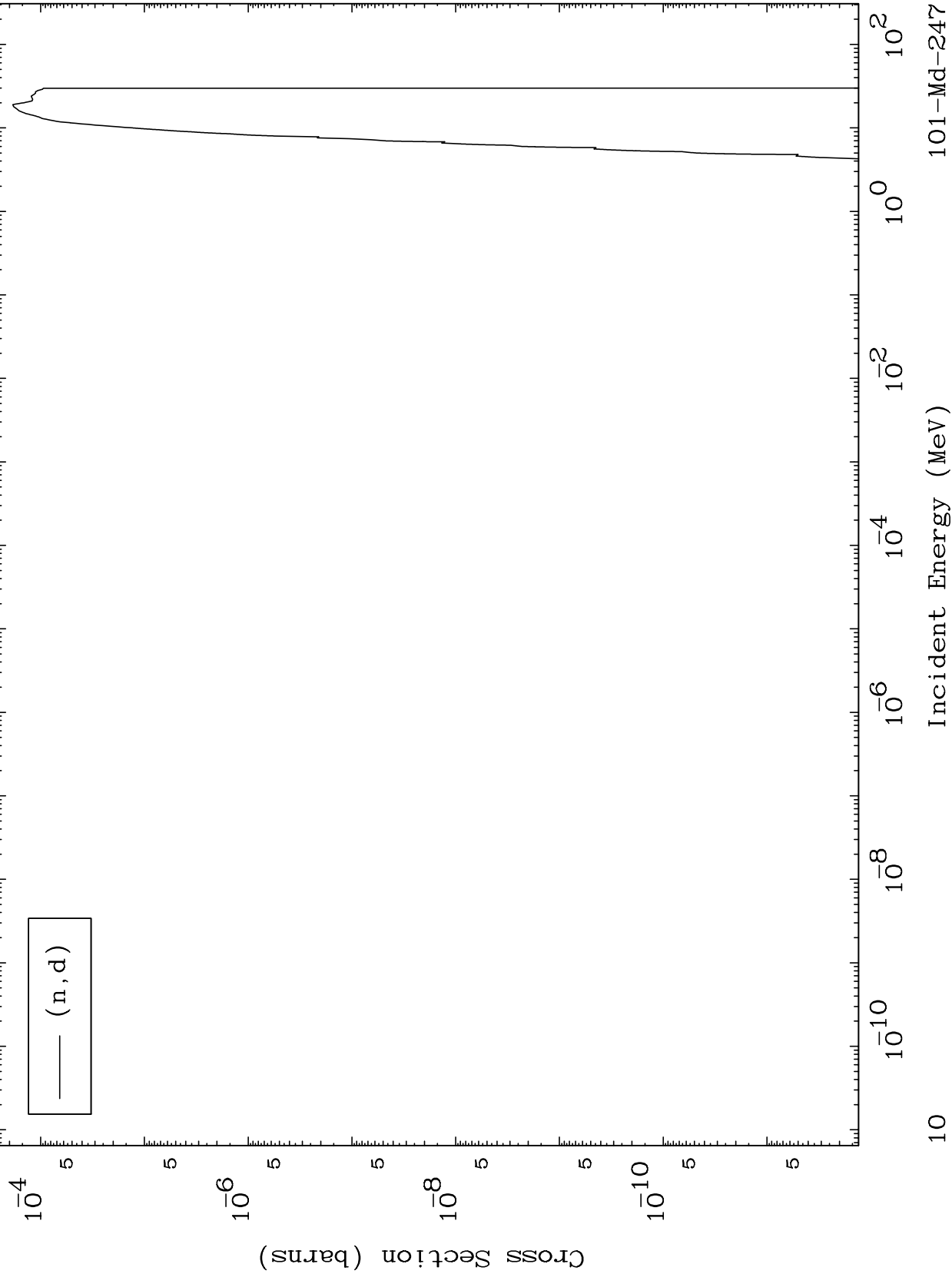
101-Md-247



MAT 9948

(n,d) Levels
293 Kelvin Cross Sections

101-Md-247



10

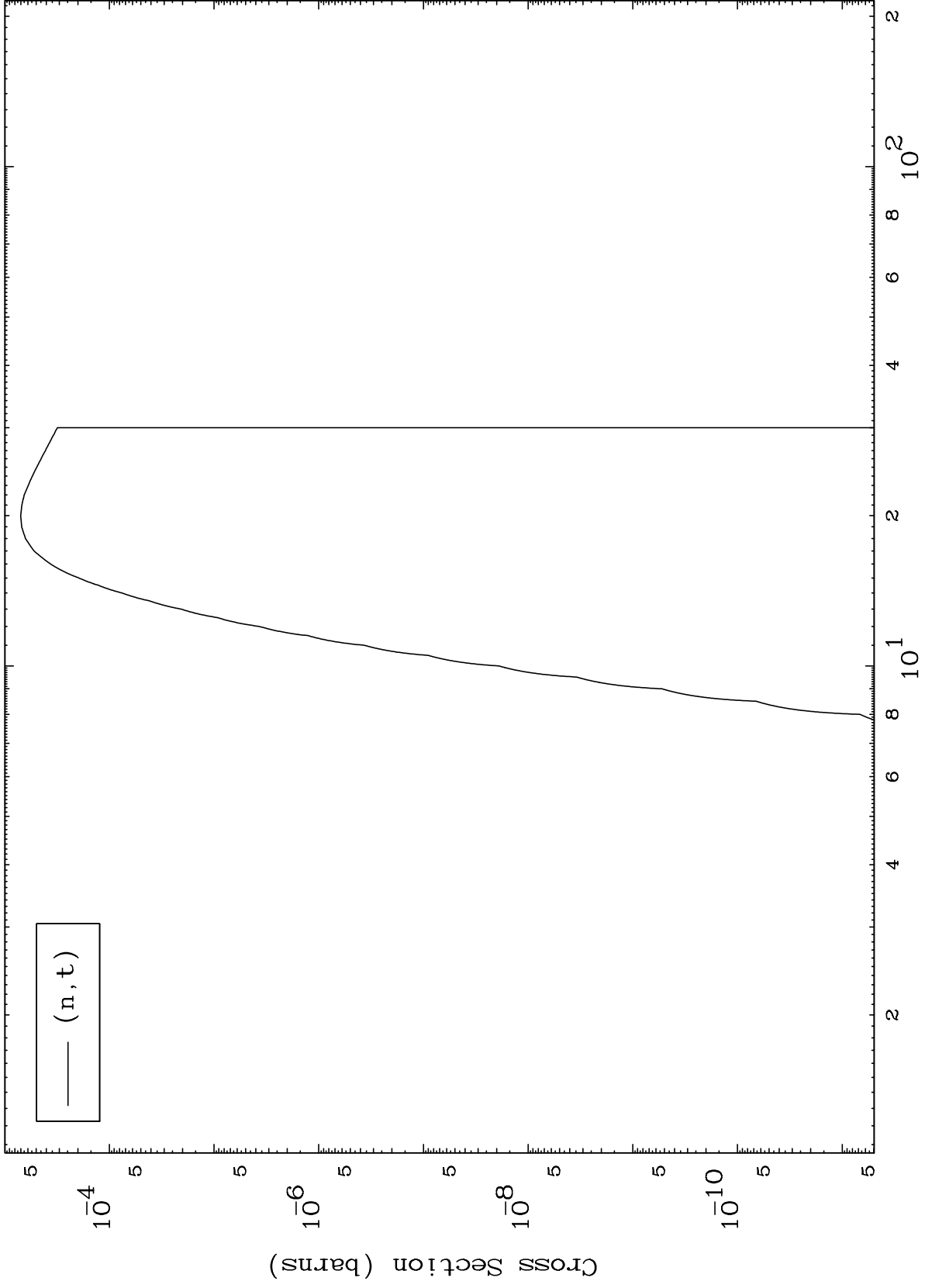
Incident Energy (MeV)

101-Md-247

MAT 9948

(n,t) Levels
293 Kelvin Cross Sections

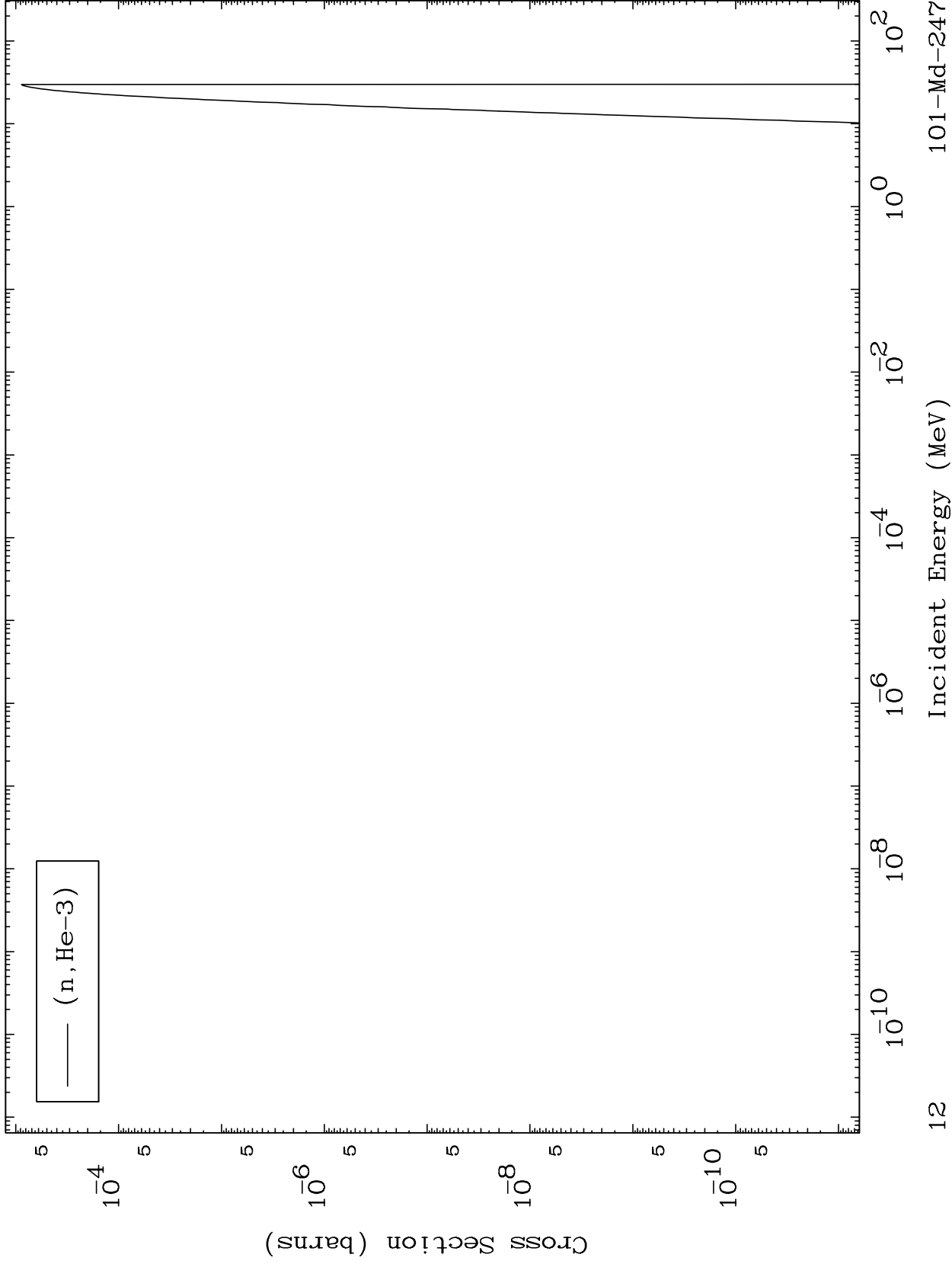
101-Md-247



MAT 9948

(n,He3) Levels
293 Kelvin Cross Sections

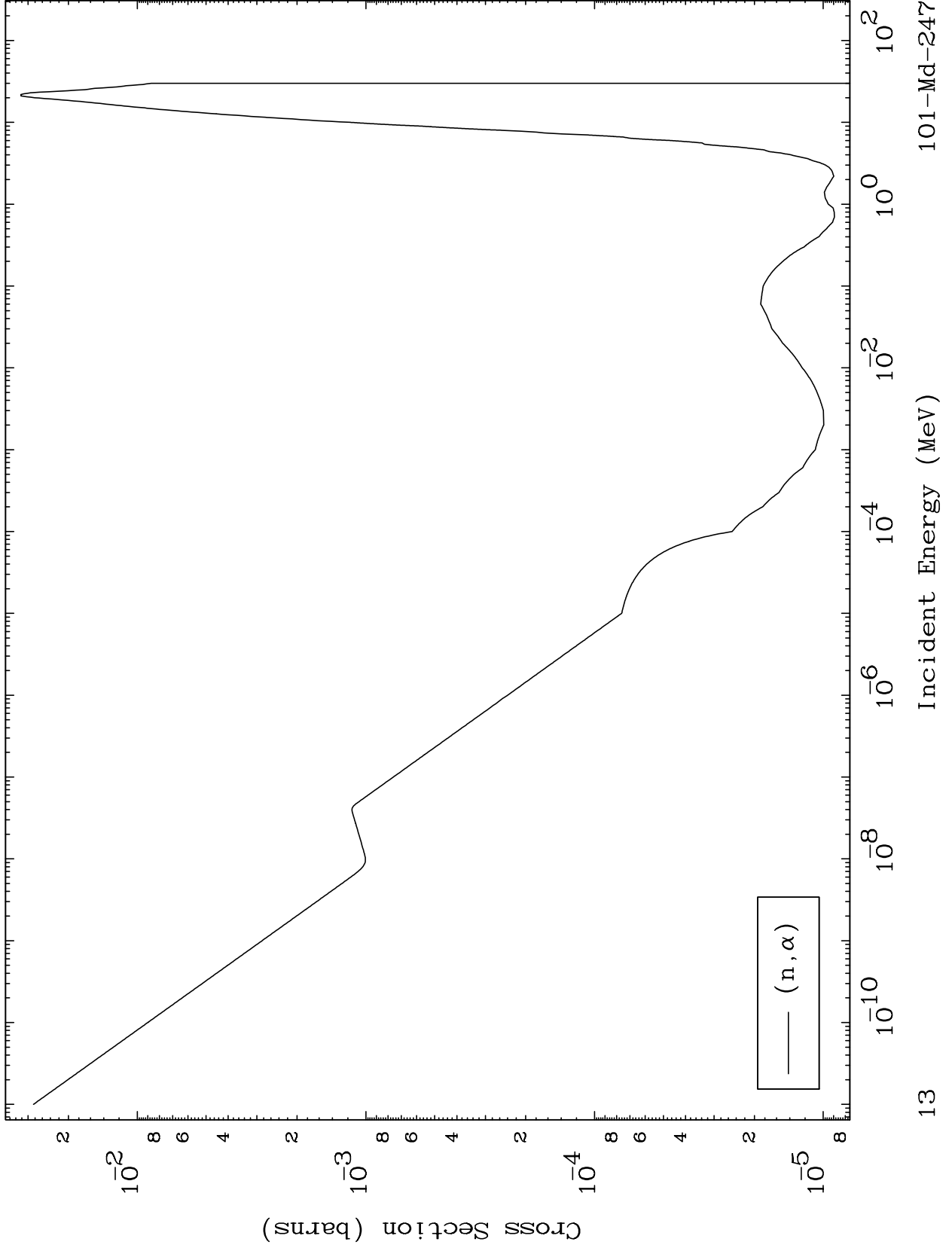
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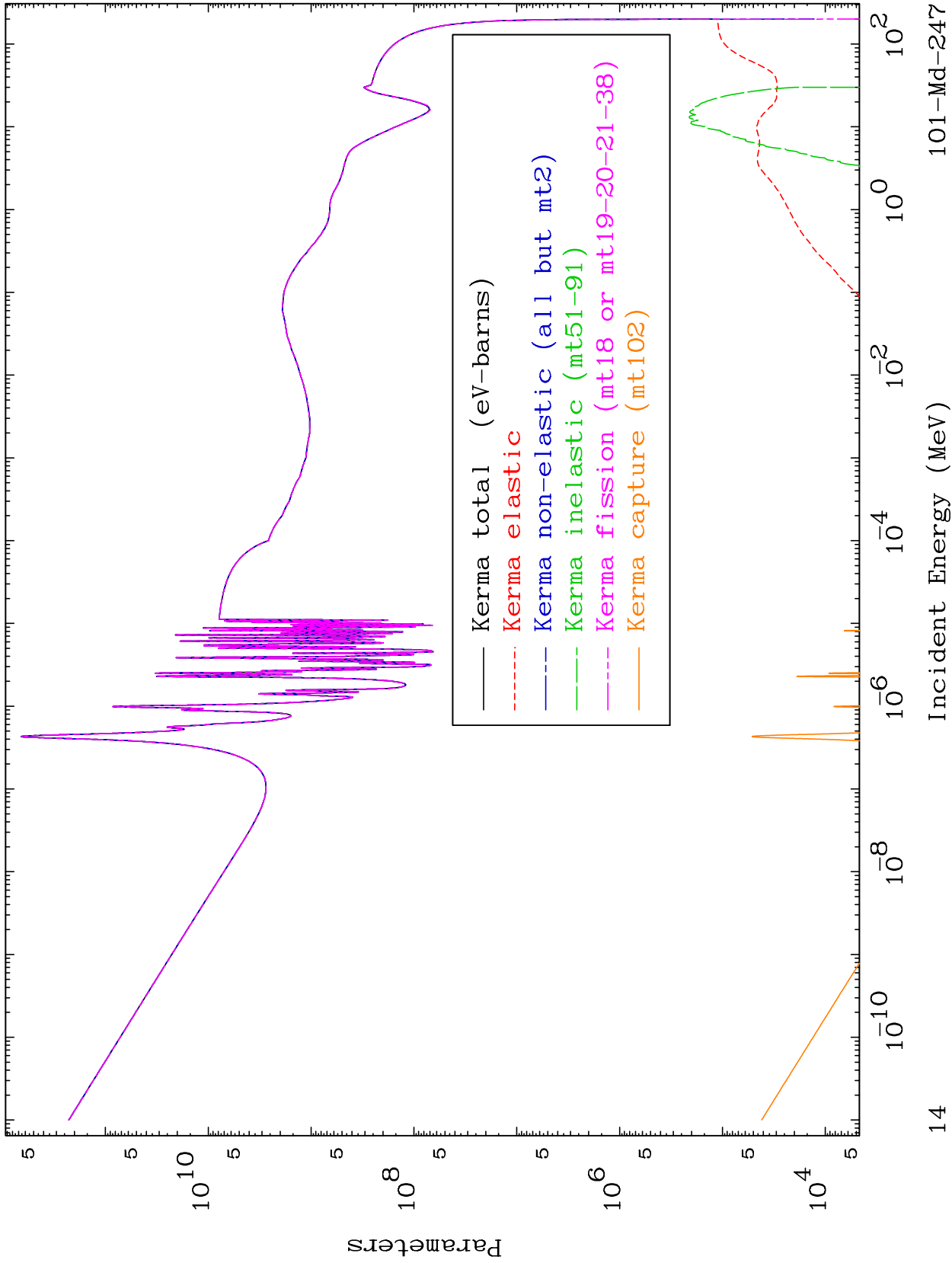


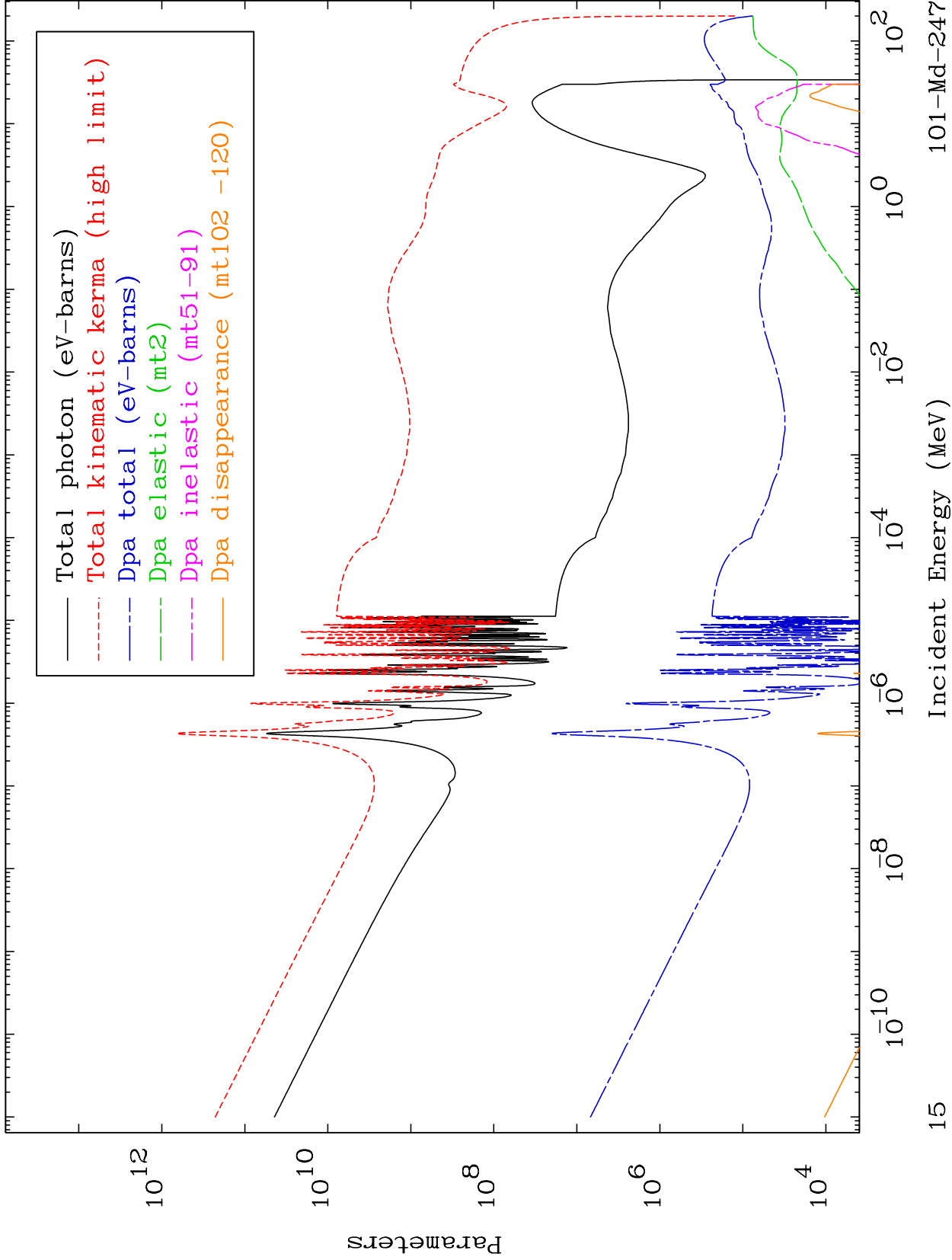
MAT 9948

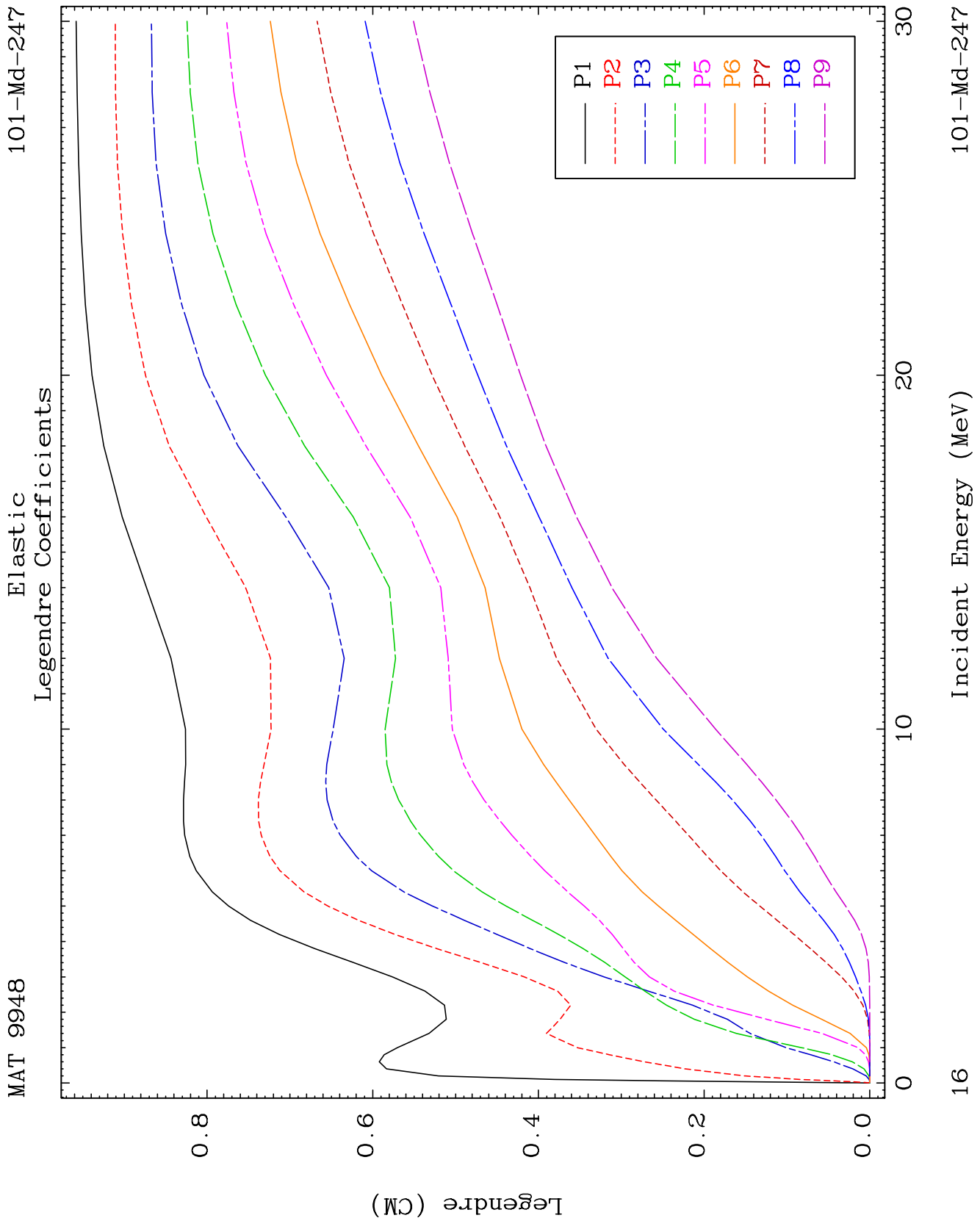
(n, α) Levels
293 Kelvin Cross Sections

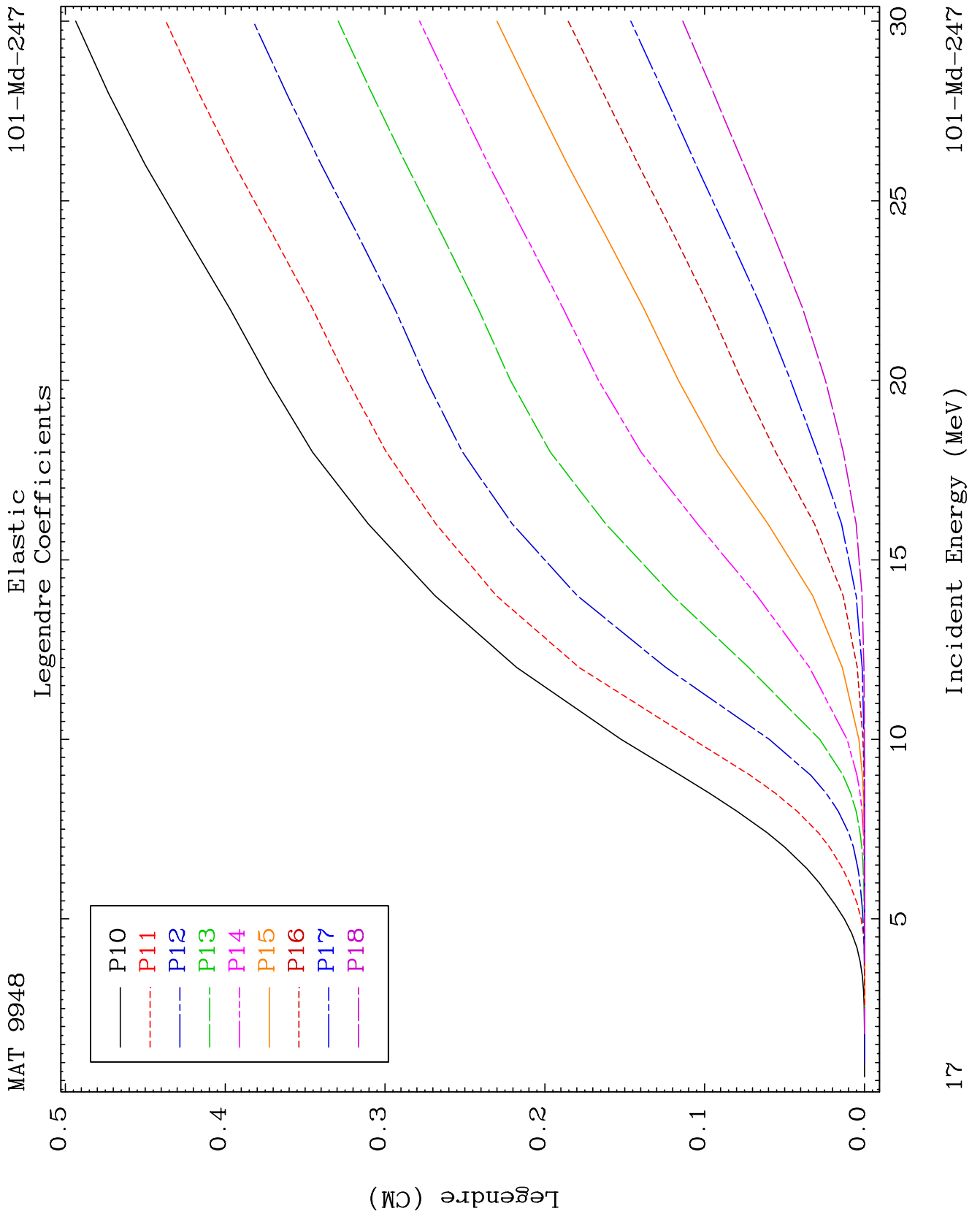
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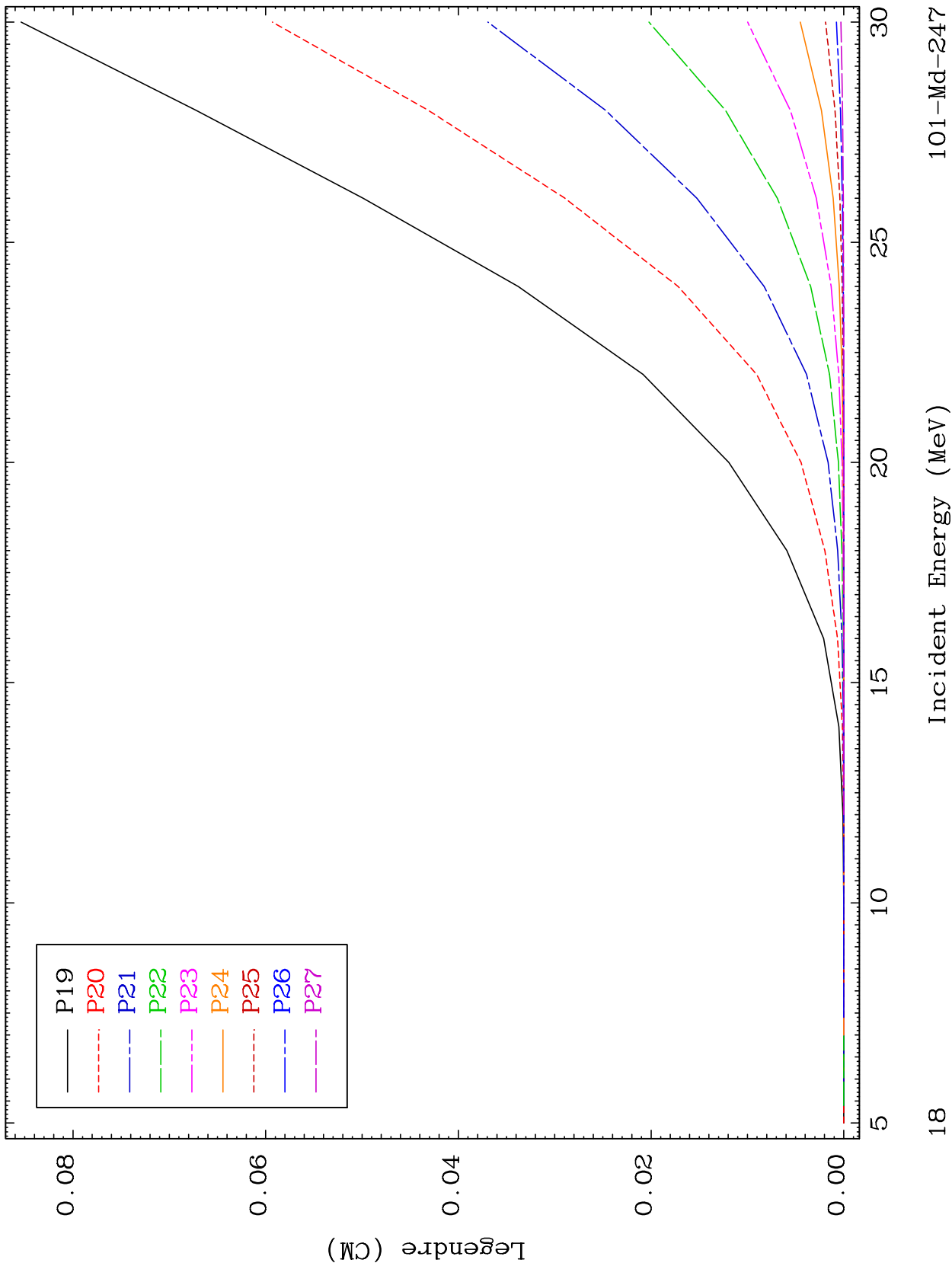




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Elastic Legendre Coefficients

101-Md-247



101-Md-247

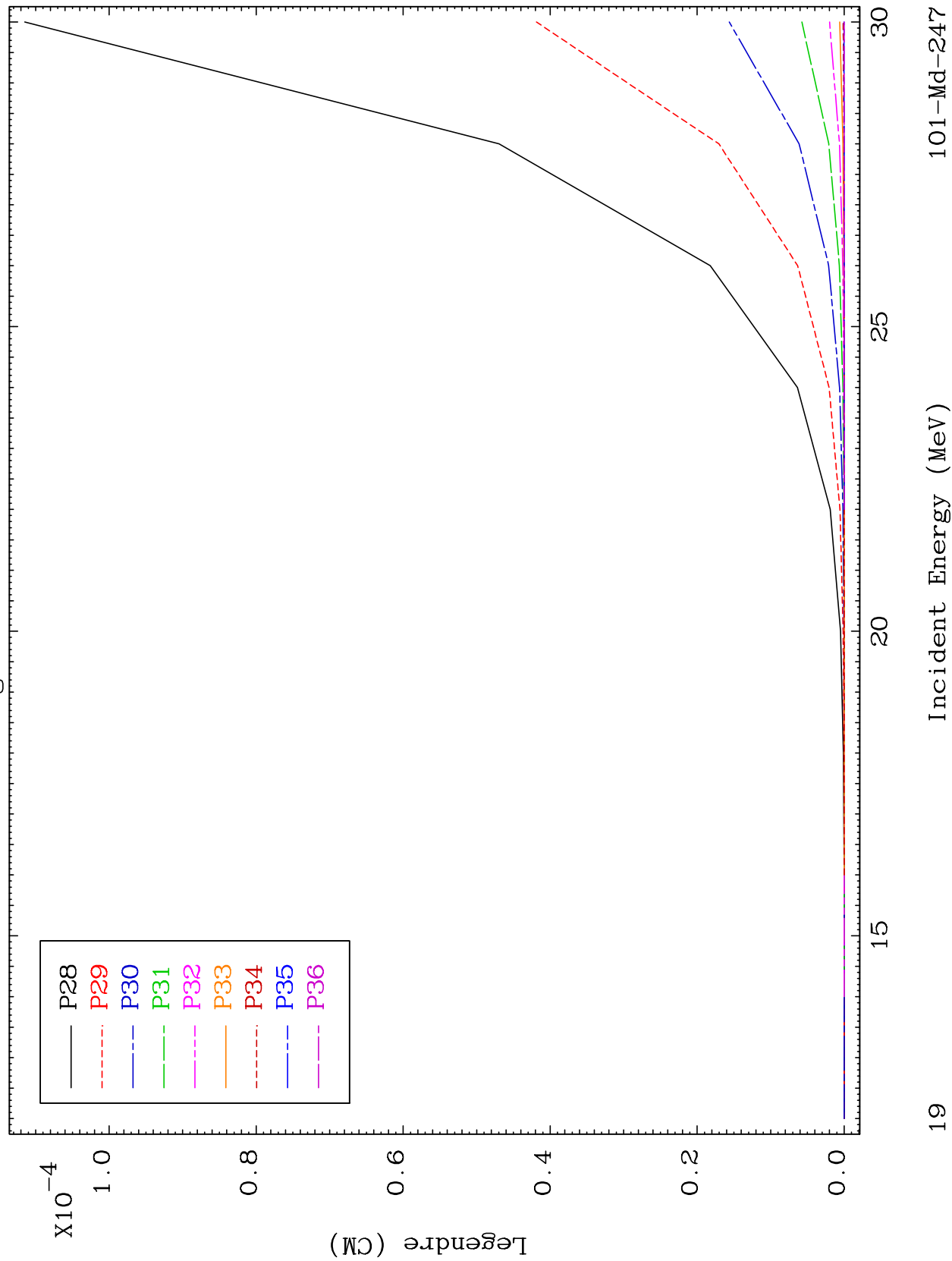
Incident Energy (MeV)

18

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Elastic Legendre Coefficients

101-Md-247



19

101-Md-247

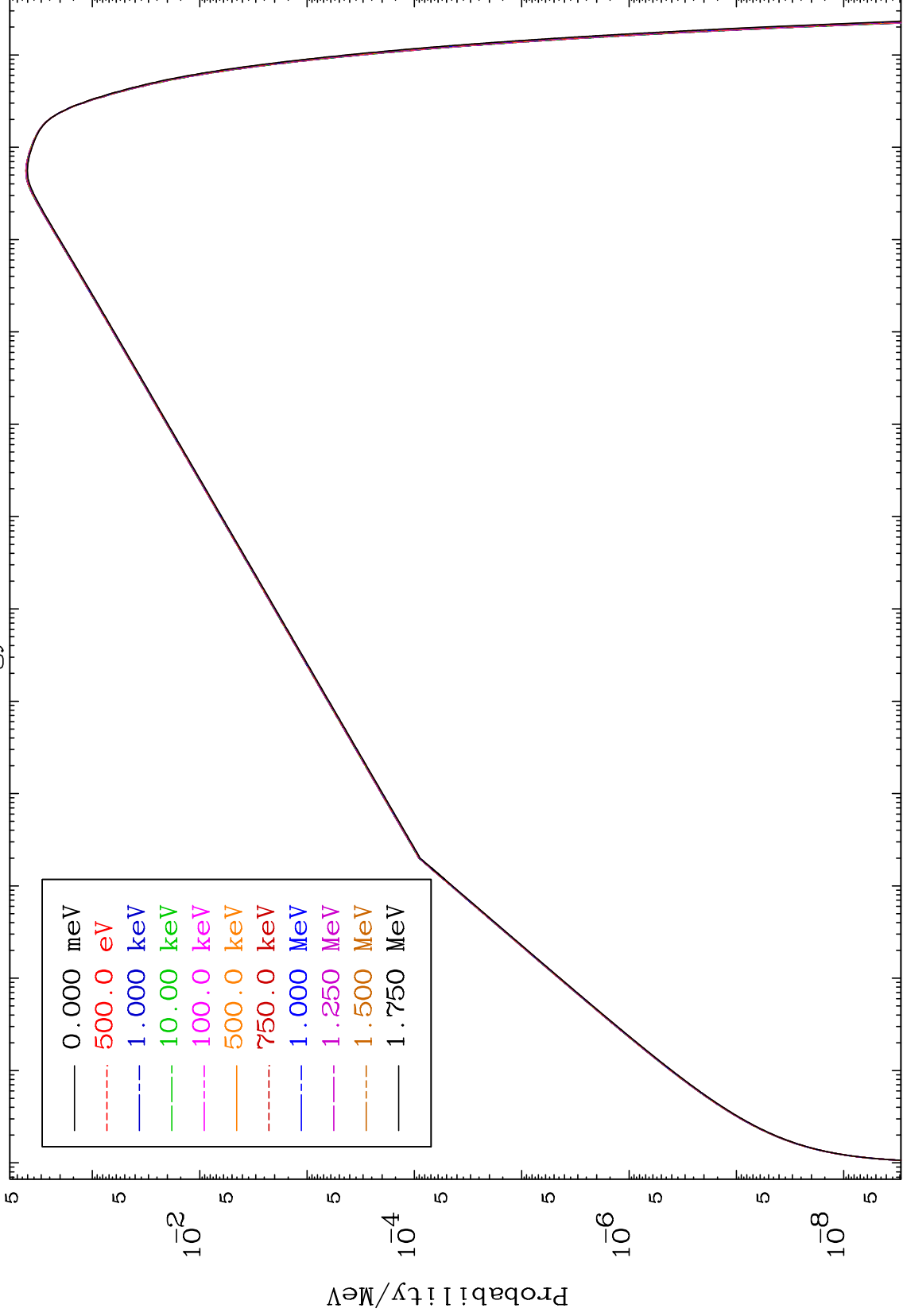
Incident Energy (MeV)

MAT 9948

Fission

101-Md-247

Energy Distribution



20

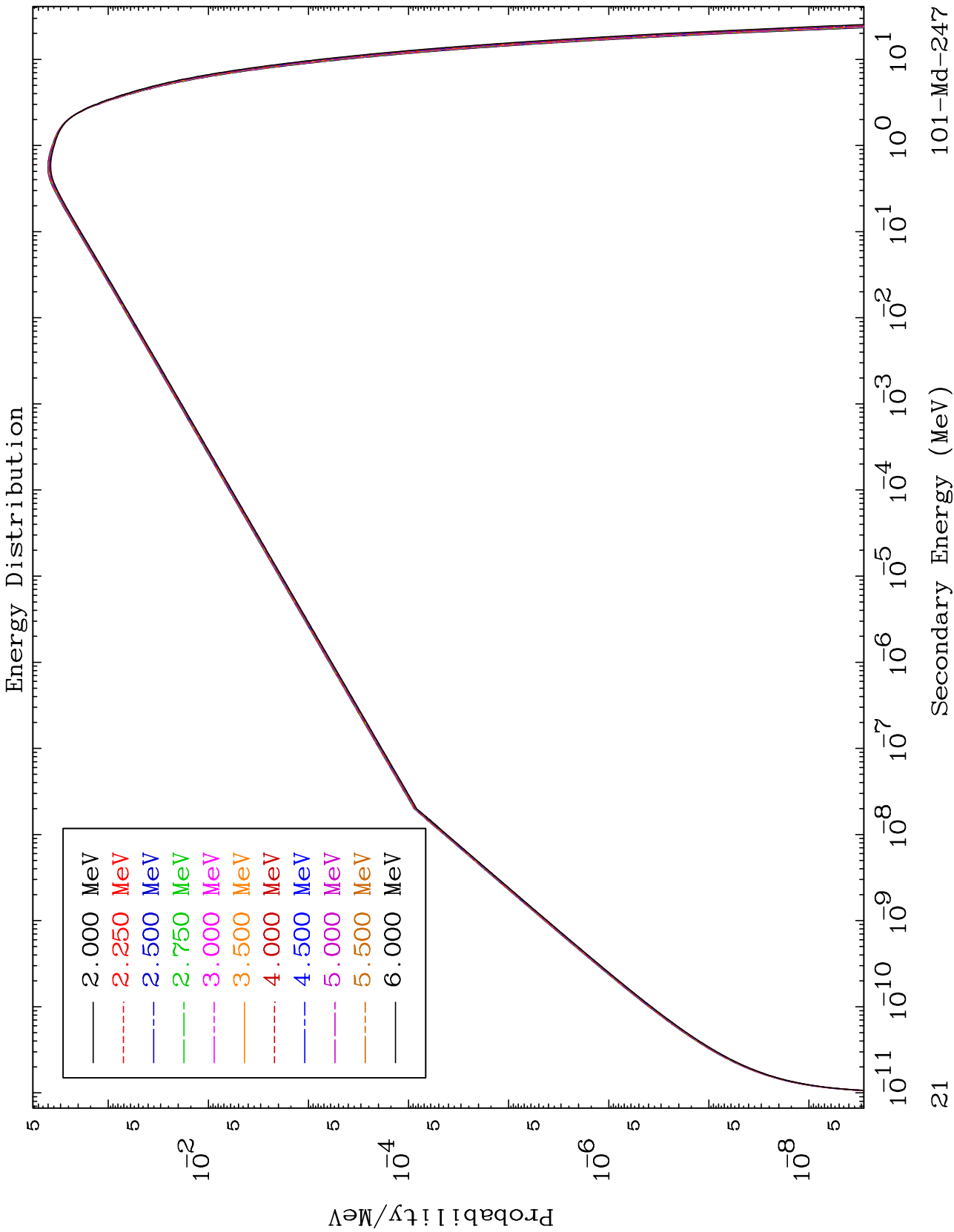
Secondary Energy (MeV)

101-Md-247

MAT 9948

Fission

101-Md-247

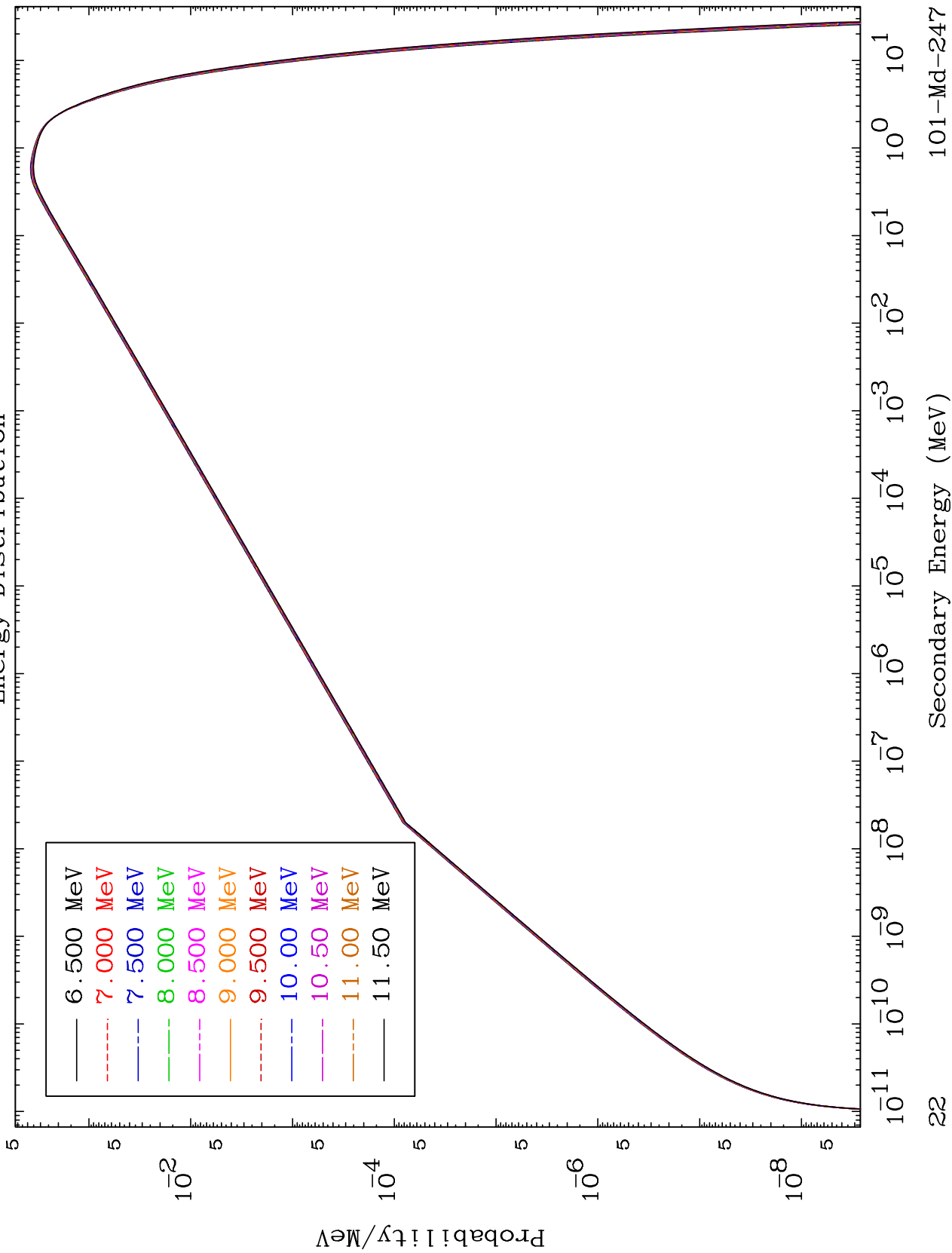


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Fission

101-Md-247

Energy Distribution



101-Md-247

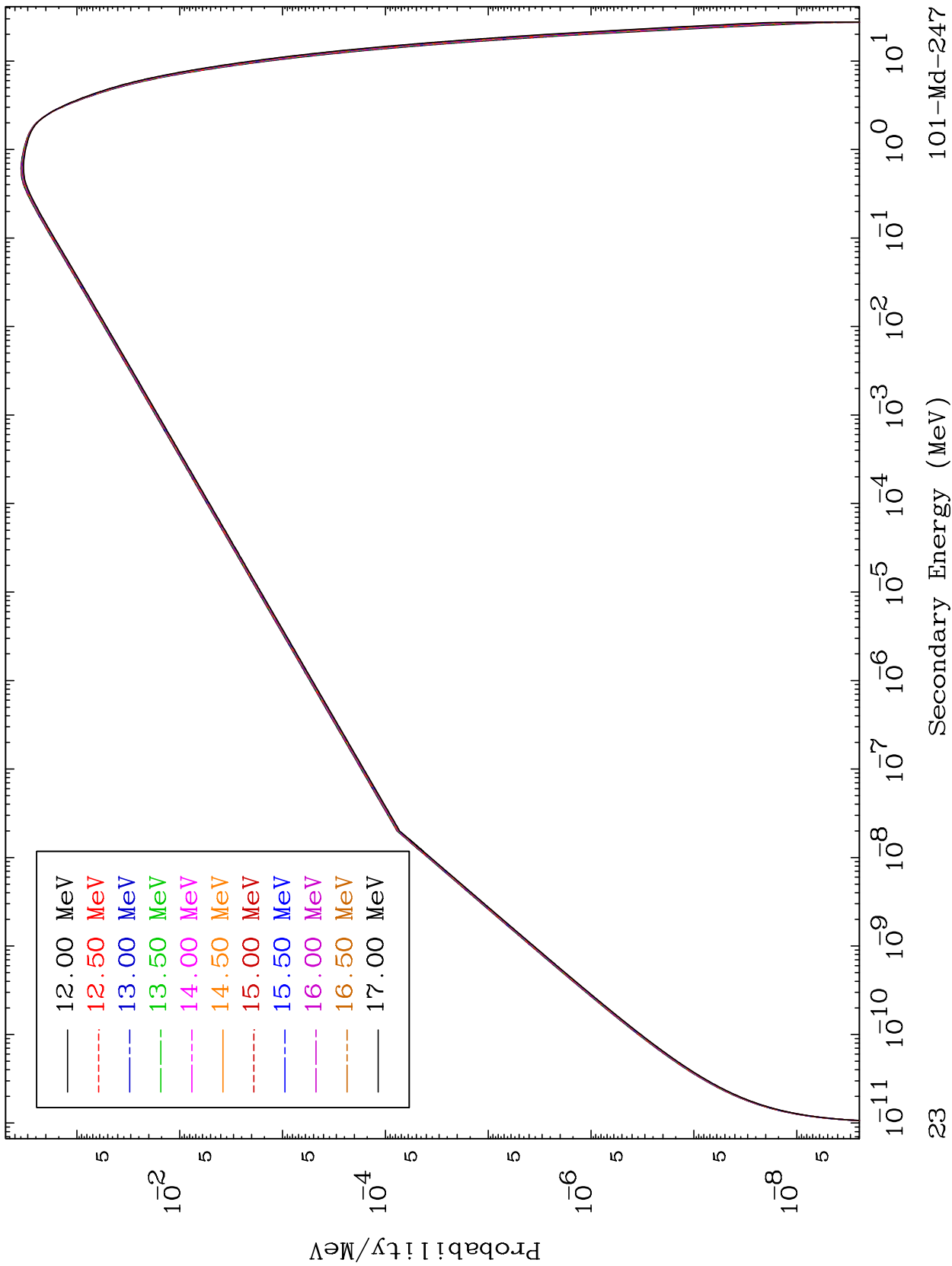
Secondary Energy (MeV)

22

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Fission Energy Distribution

101-Md-247



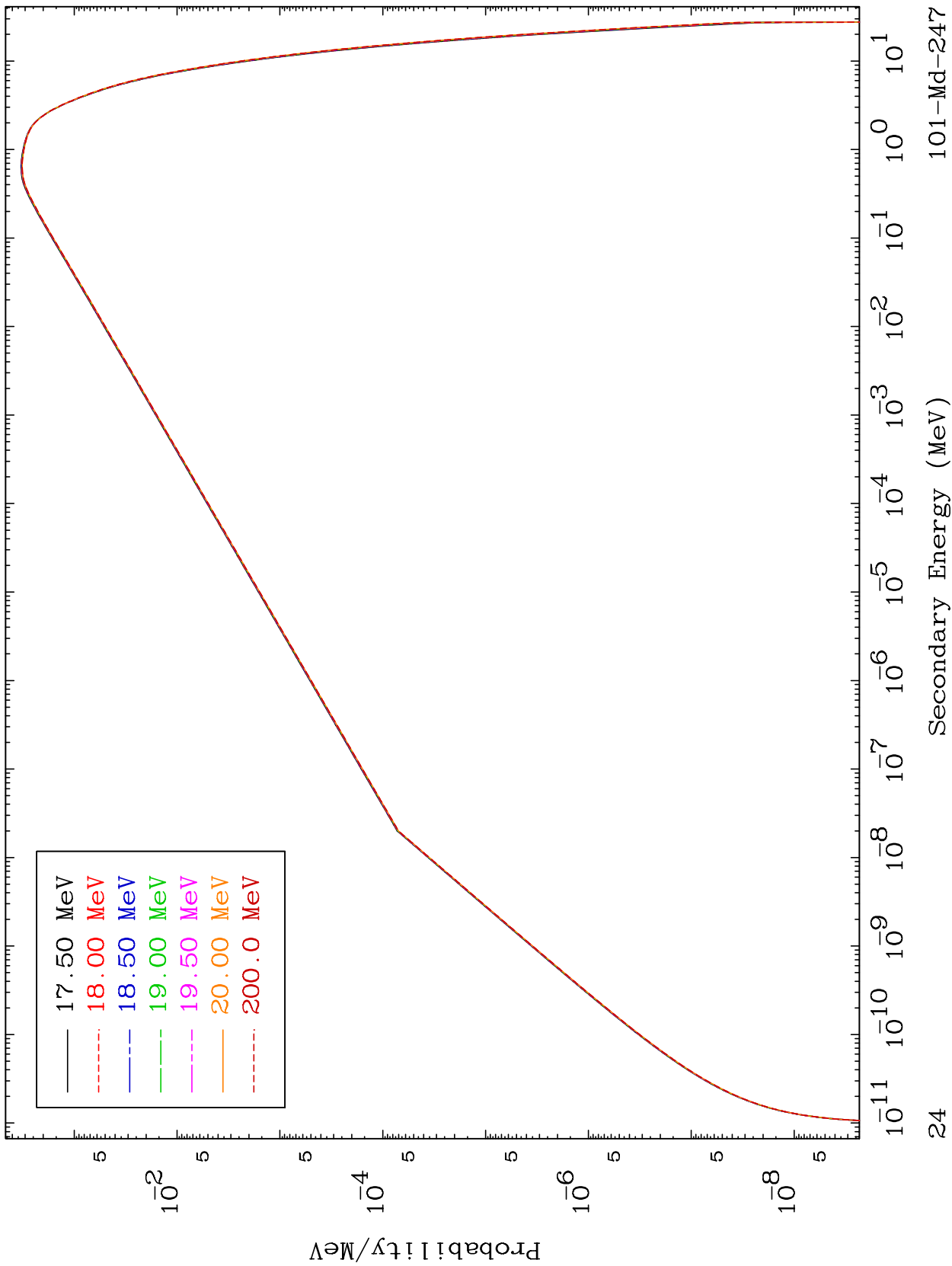
101-Md-247

Secondary Energy (MeV)

MAT 9948

Fission
Energy Distribution

101-Md-247

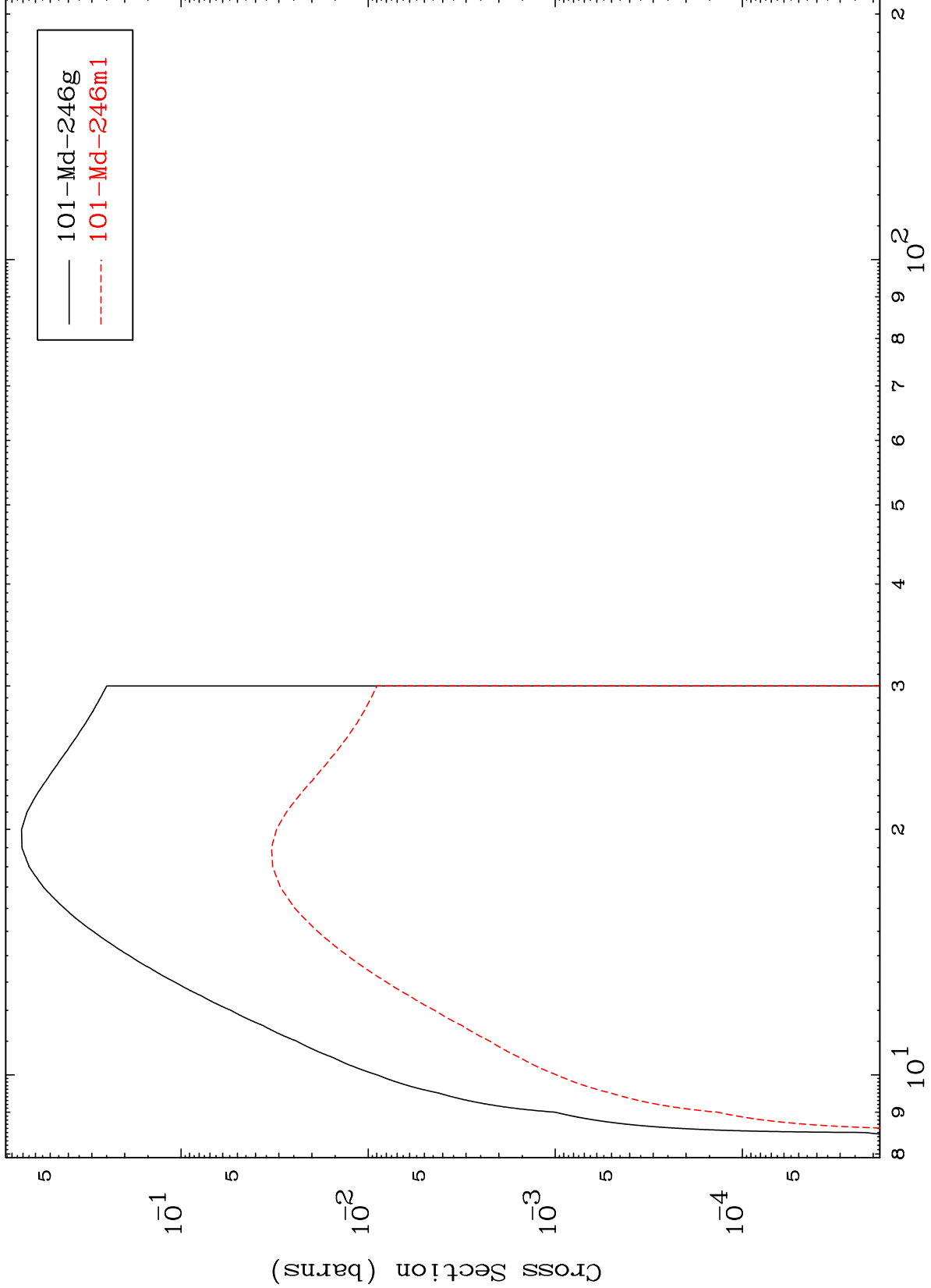


MAT 9948

(n,2n)

101-Md-247

Radionuclide Production Cross Section



101-Md-246g
101-Md-246m1

25

Incident Energy (MeV)

101-Md-247

MAT 9948

101-Md-247

(n,p)
Radionuclide Production Cross Section

