

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

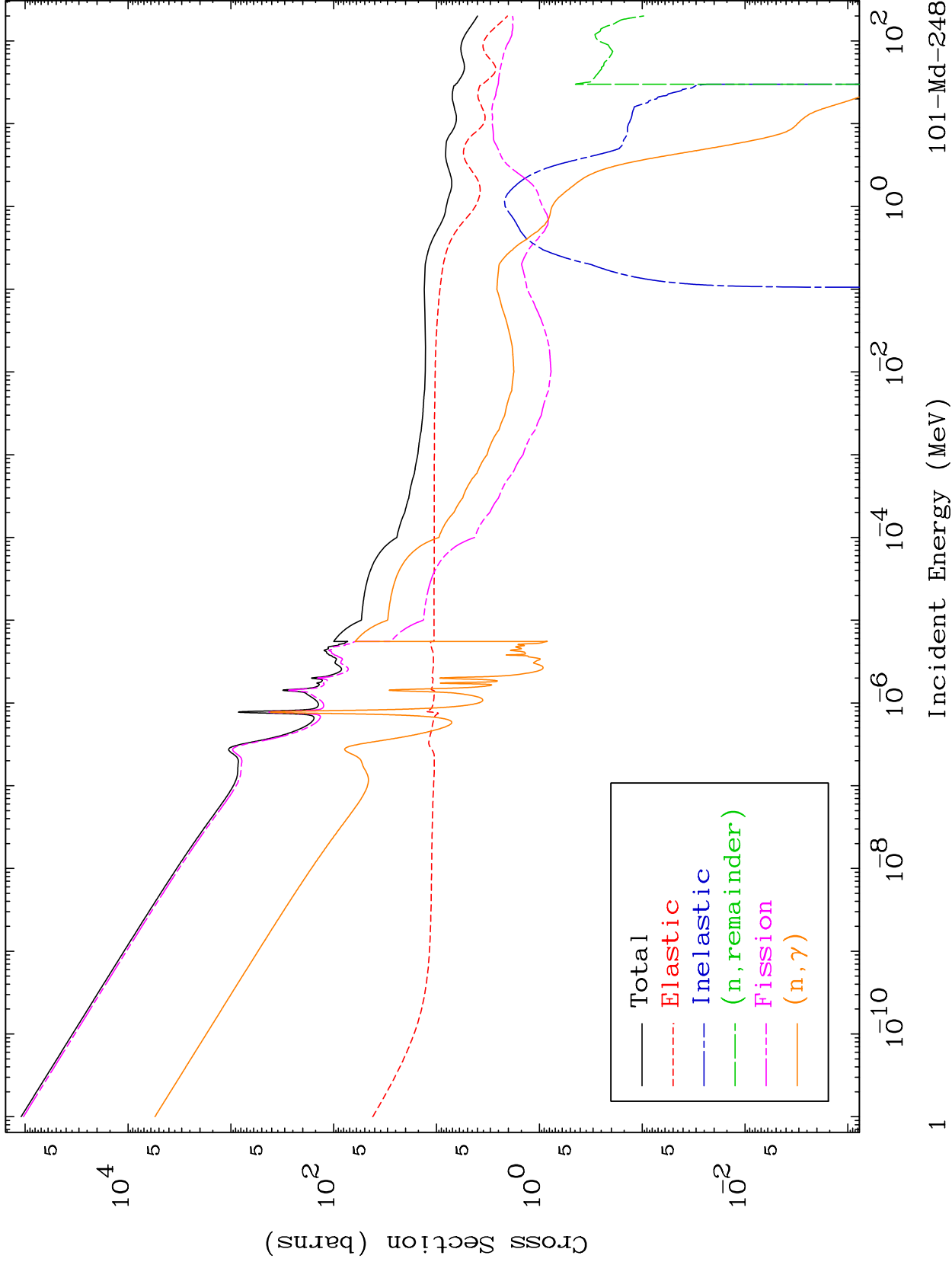
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Press Mouse Button to Start

MAT 9949

Major
293 Kelvin Cross Sections

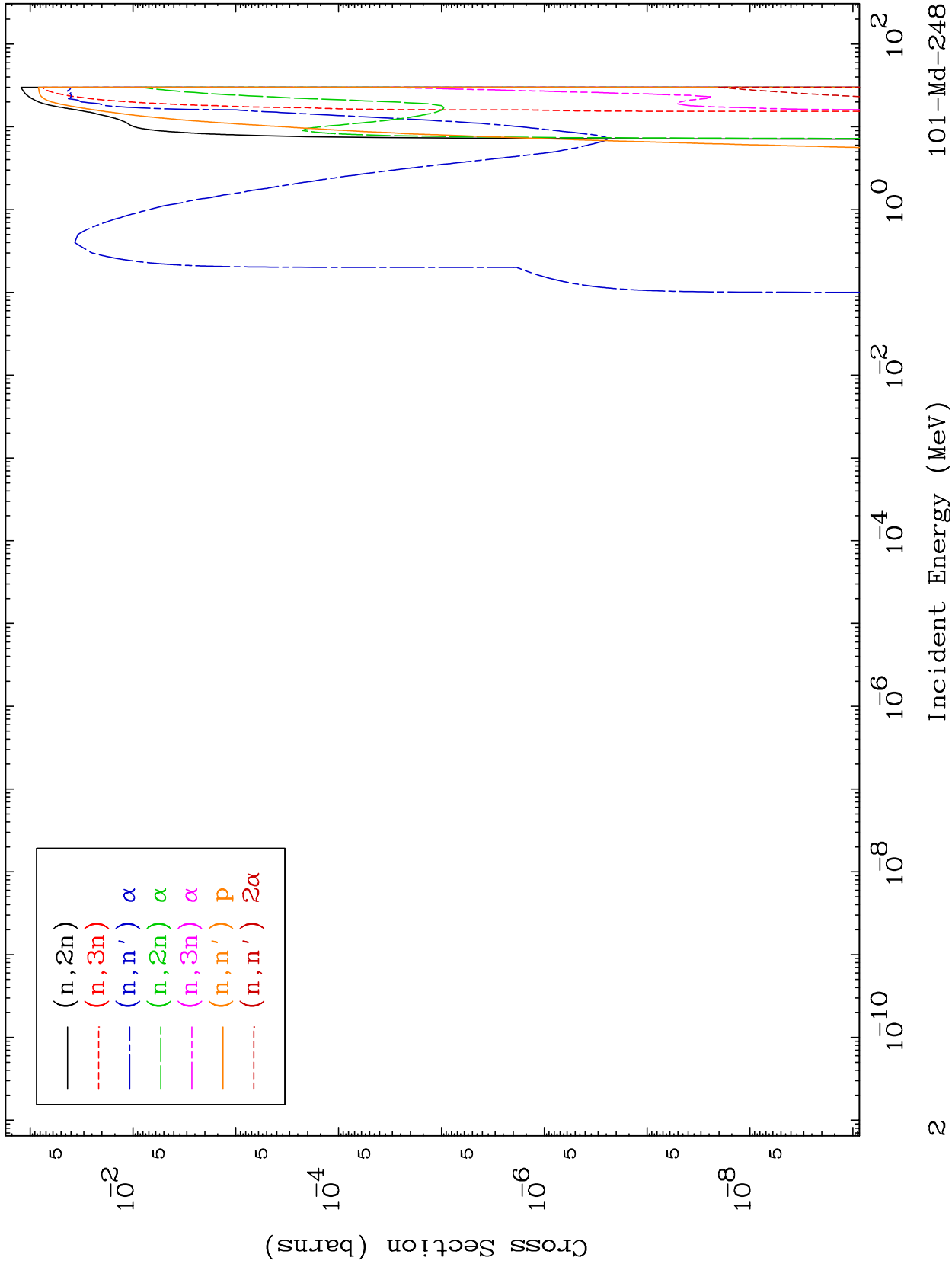
101-Md-248

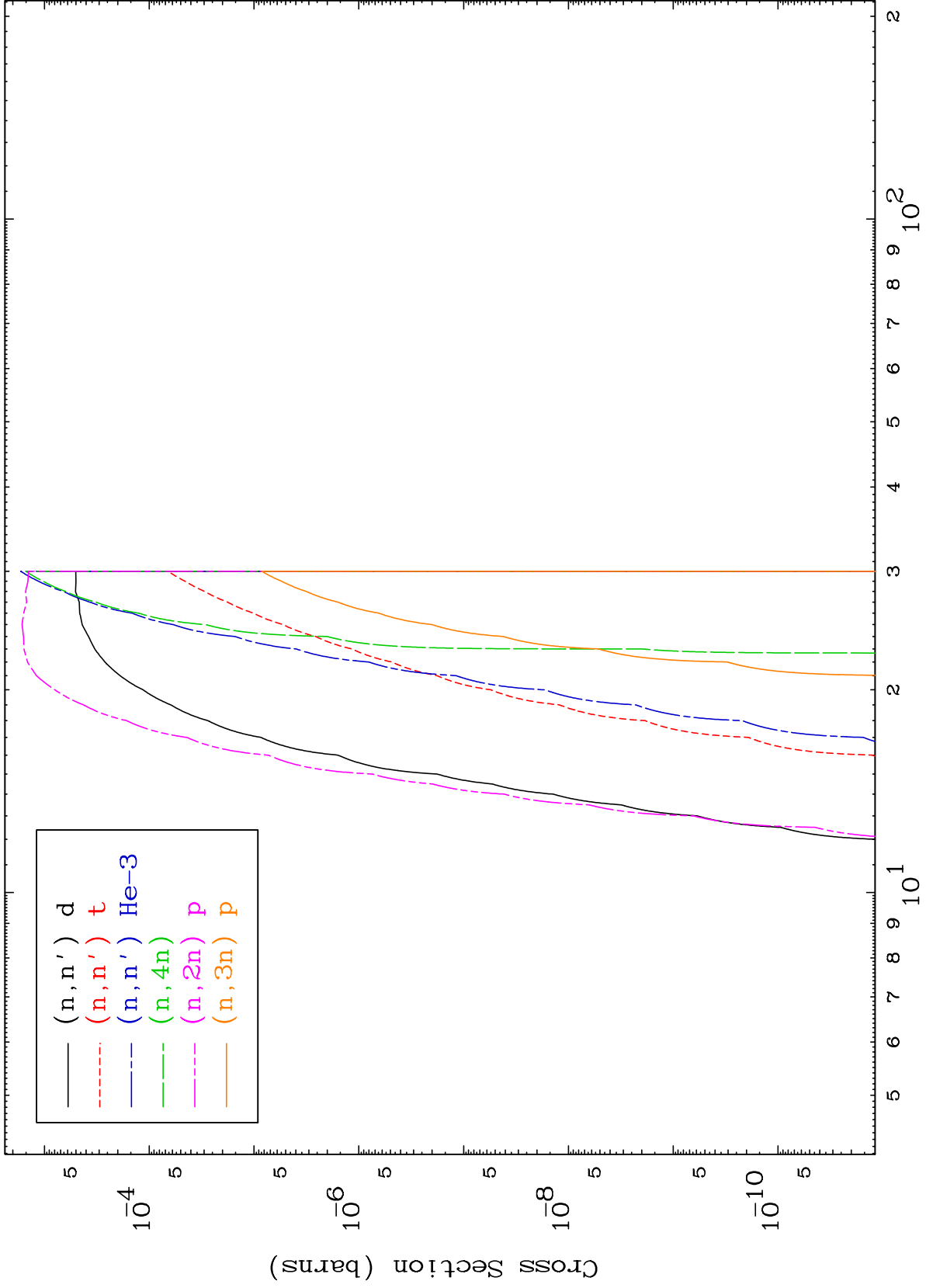


MAT 9949

Neutron Production
293 Kelvin Cross Sections

101-Md-248

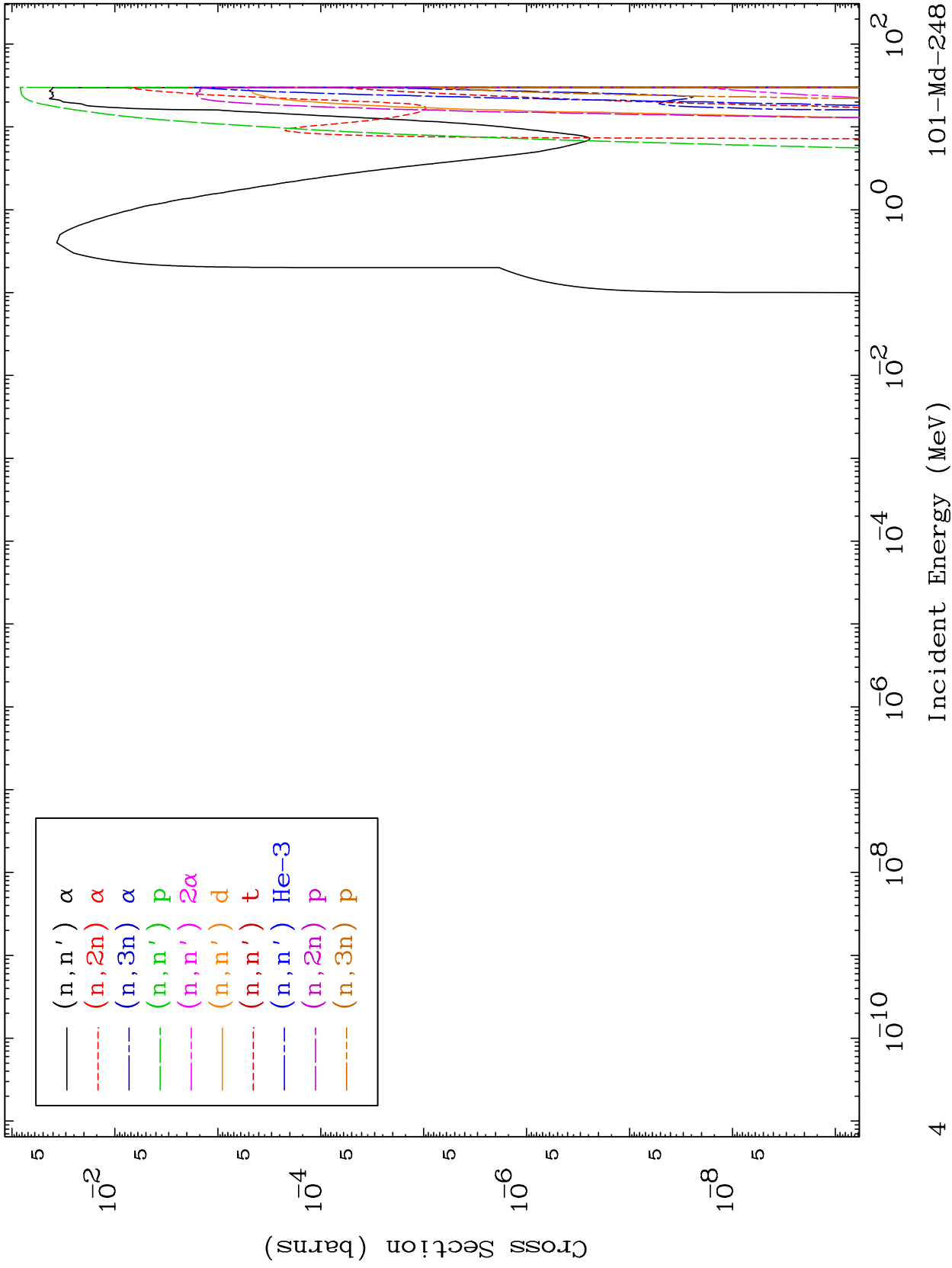




MAT 9949

Charged Particle
293 Kelvin Cross Sections

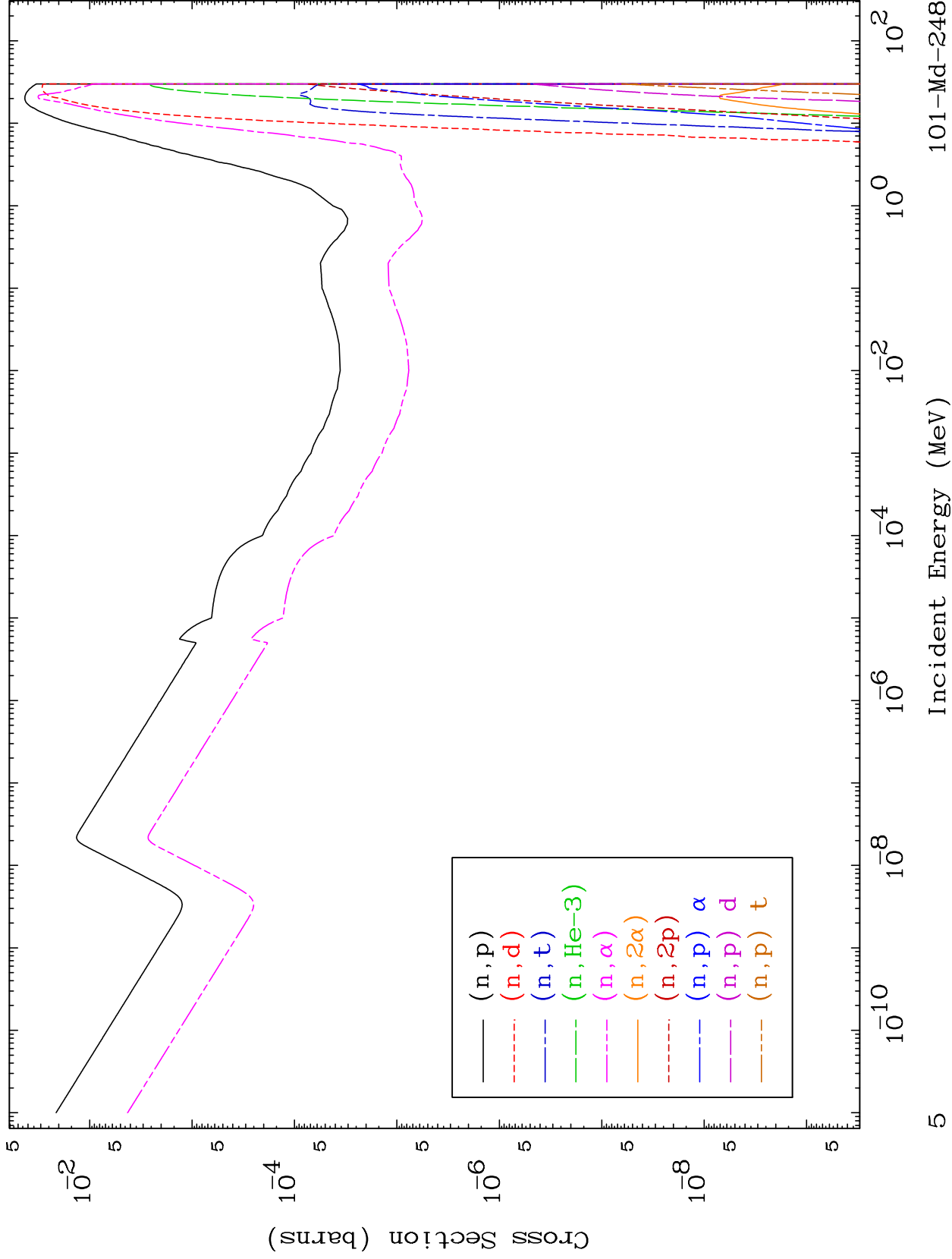
101-Md-248



MAT 9949

Charged Particle
293 Kelvin Cross Sections

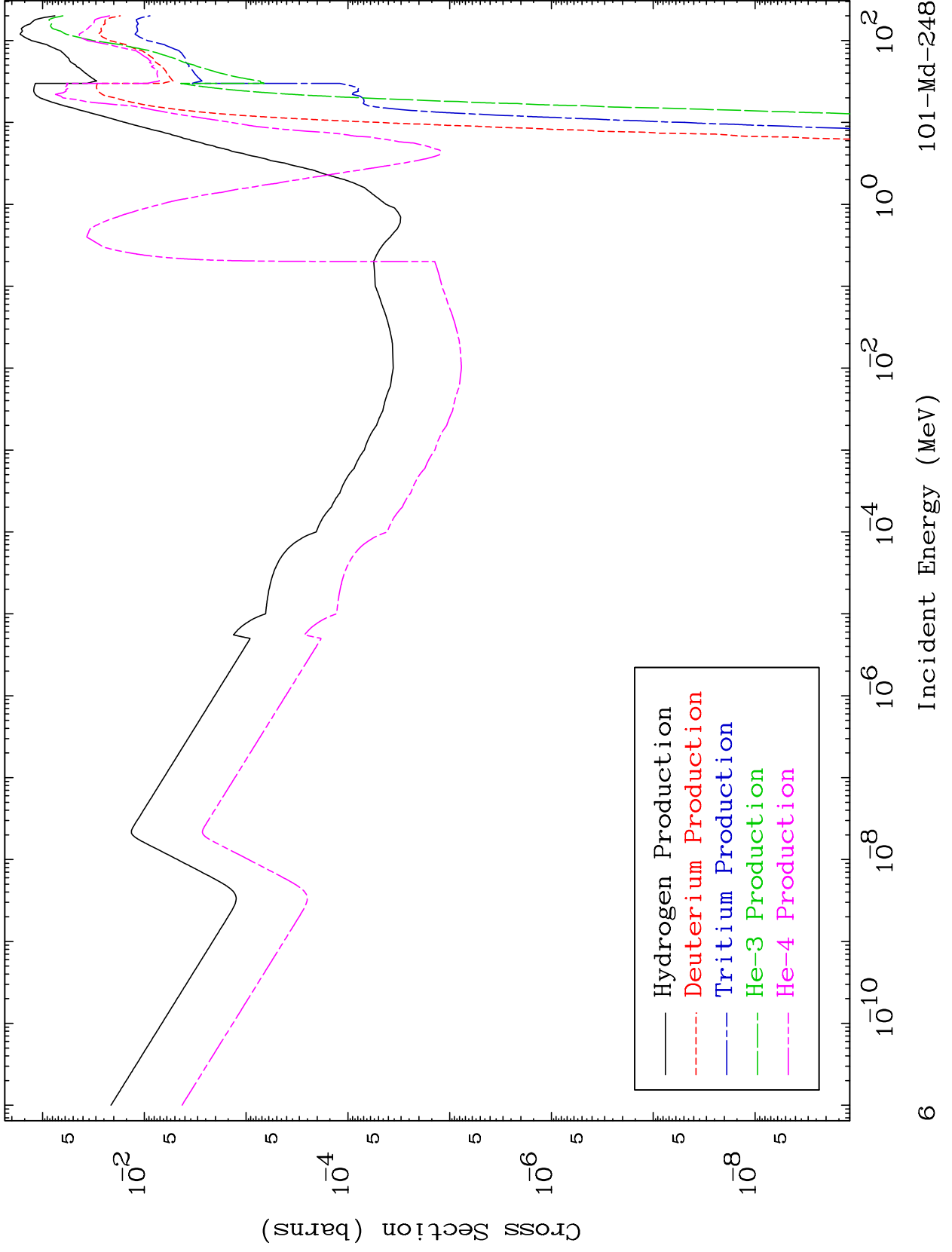
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MAT 9949

Particle Production
293 Kelvin Cross Sections

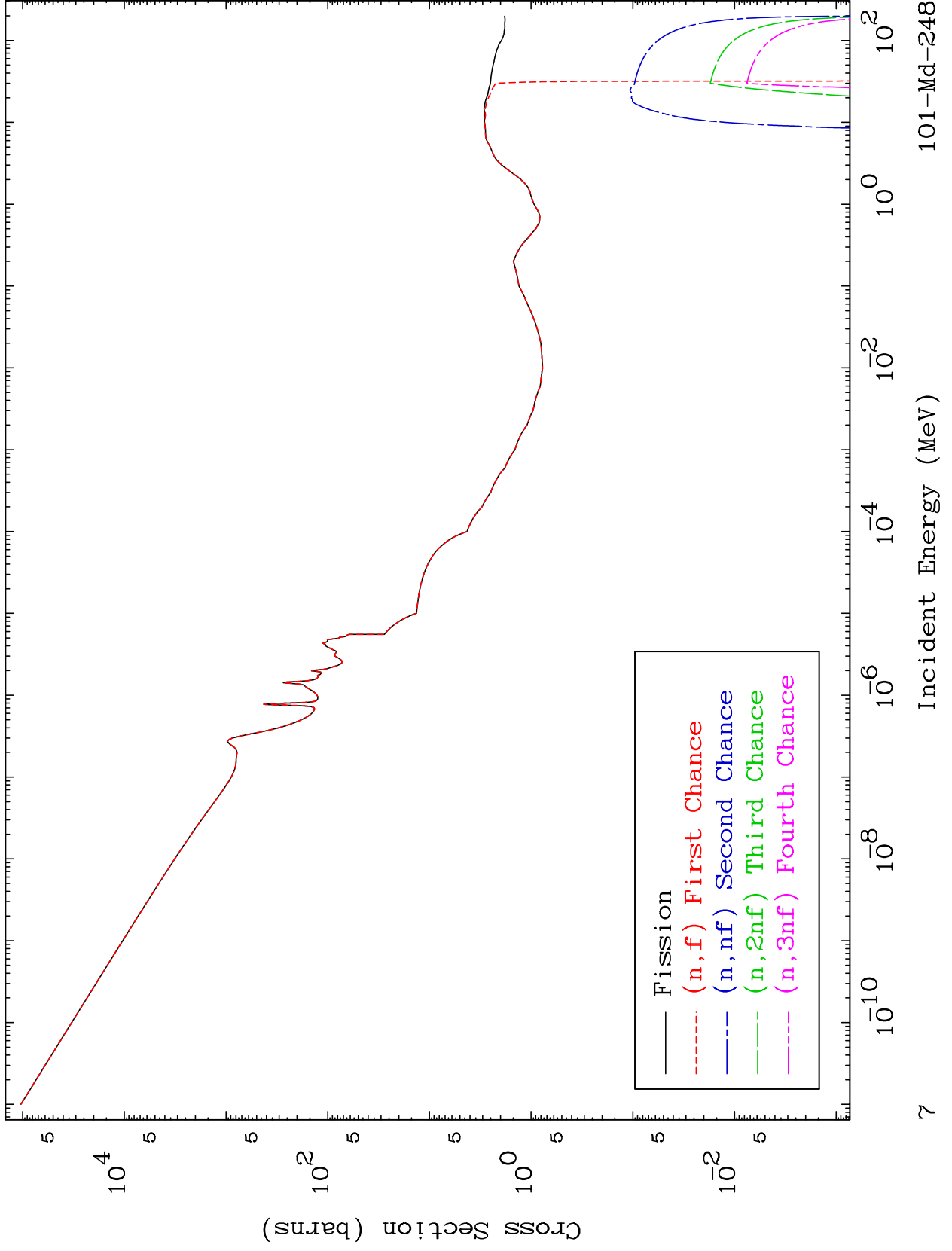
101-Md-248



MAT 9949

Fission
293 Kelvin Cross Sections

101-Md-248

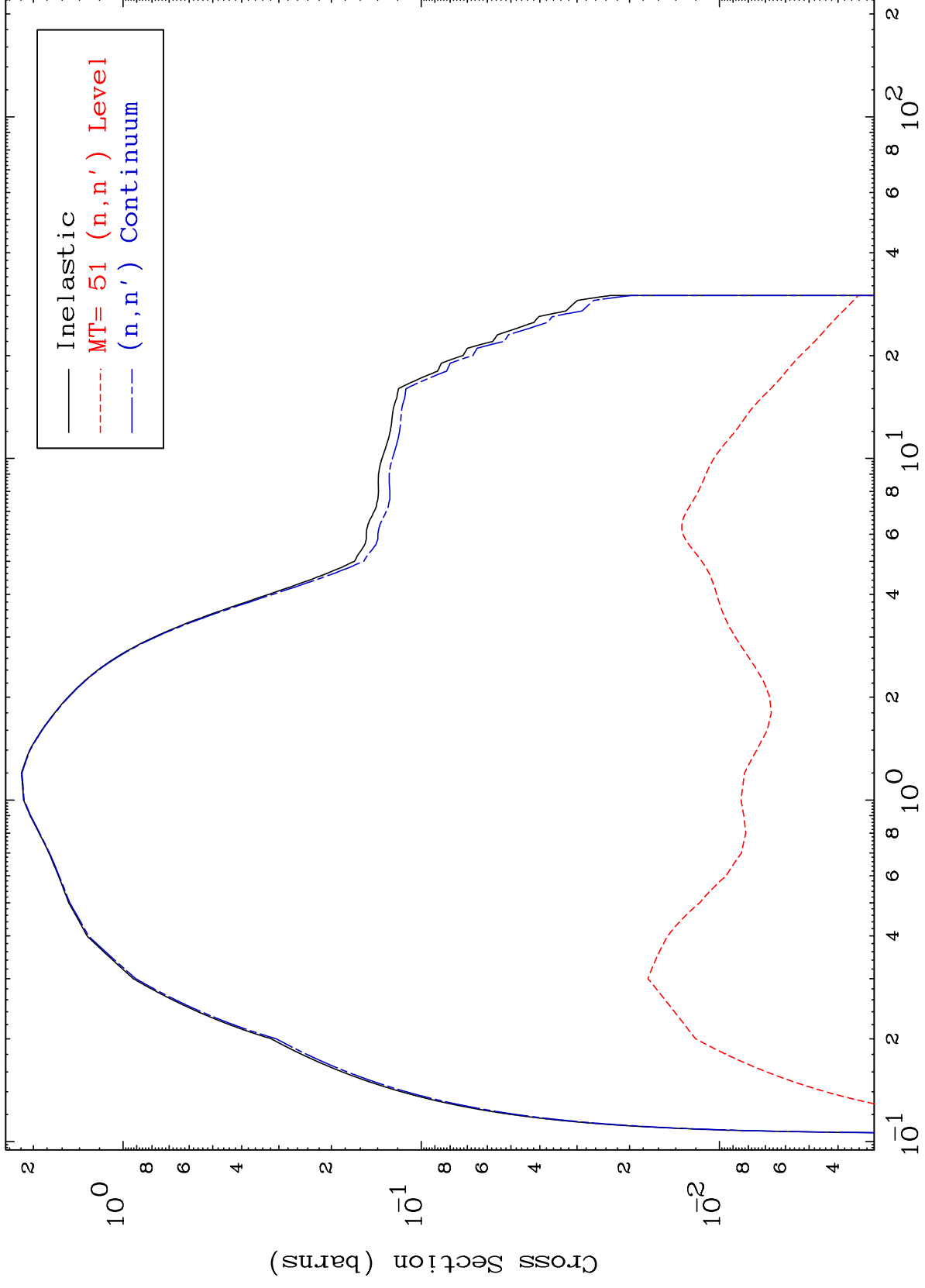


MAT 9949

(n,n') Level

101-Md-248

293 Kelvin Cross Sections



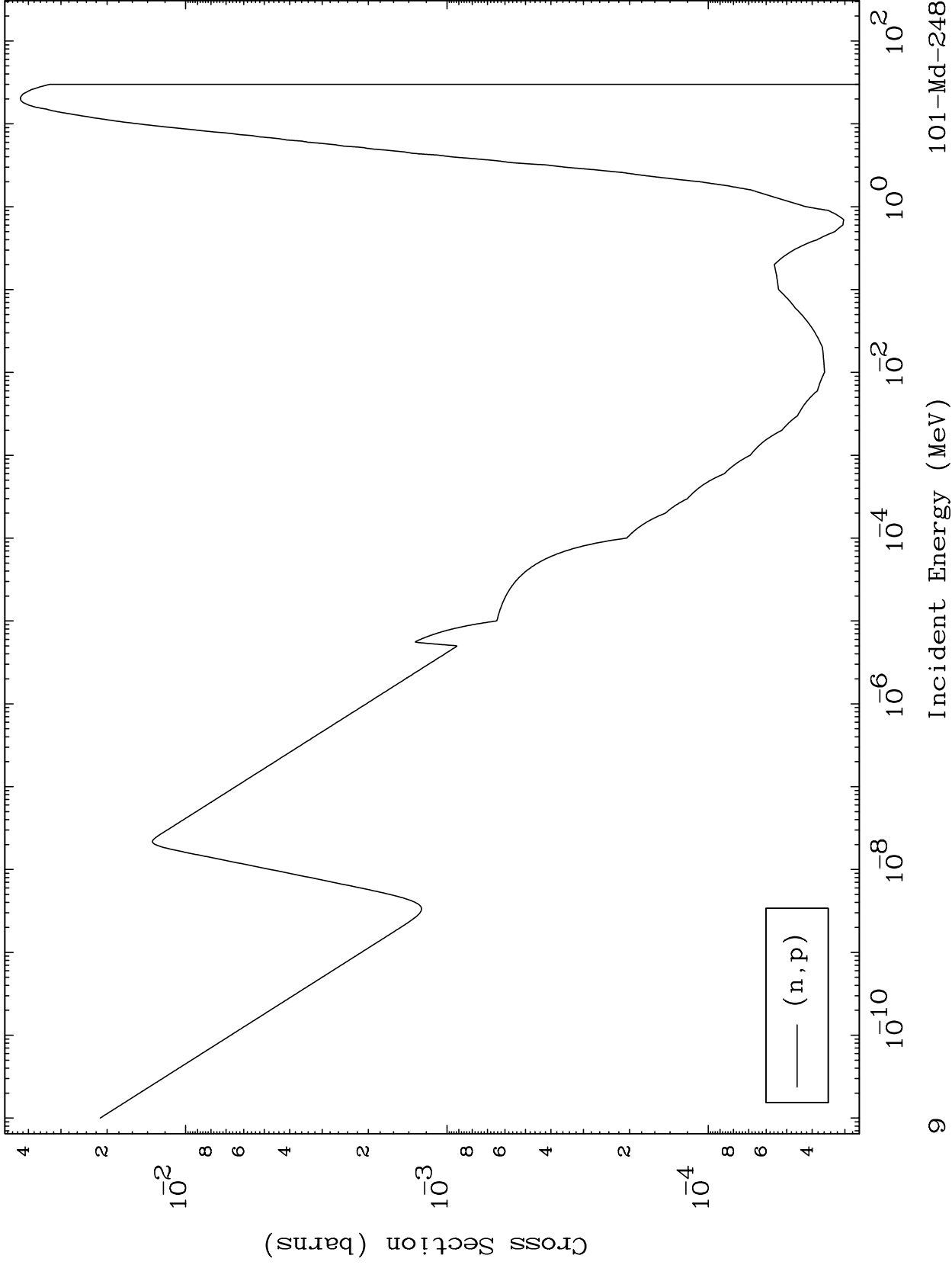
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,p) Levels
293 Kelvin Cross Sections

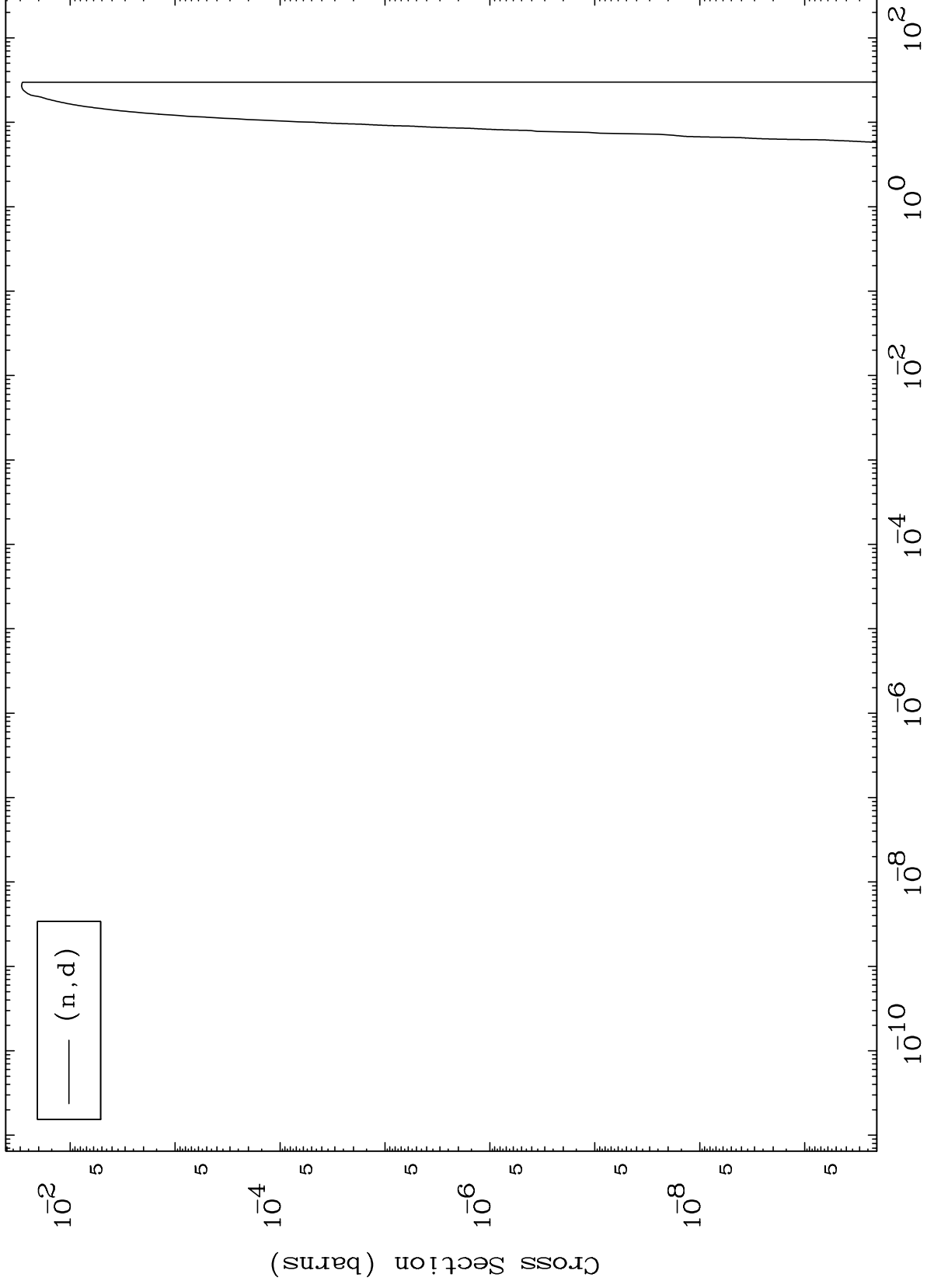
101-Md-248



MAT 9949

(n,d) Levels
293 Kelvin Cross Sections

101-Md-248



10

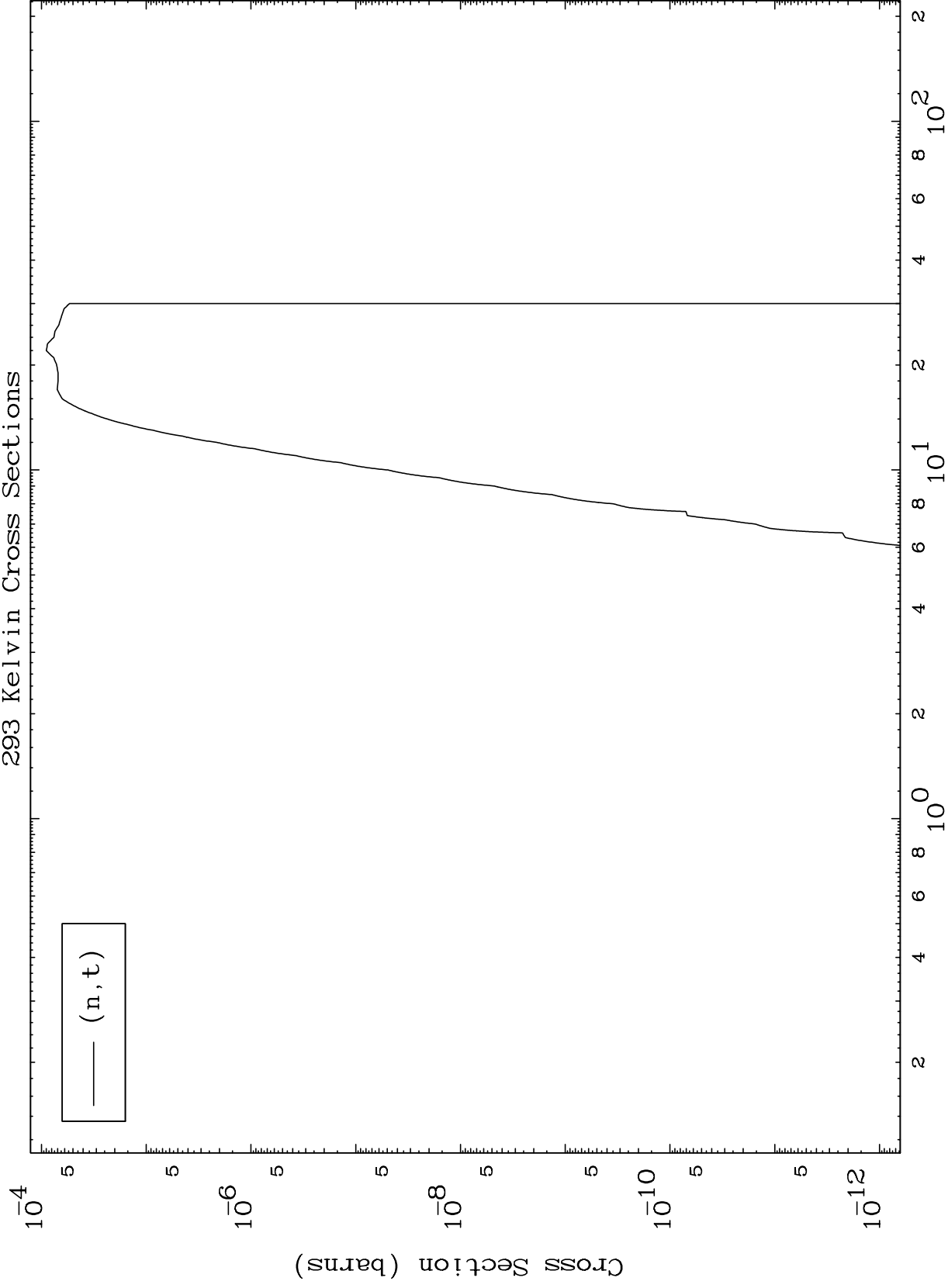
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,t) Levels
293 Kelvin Cross Sections

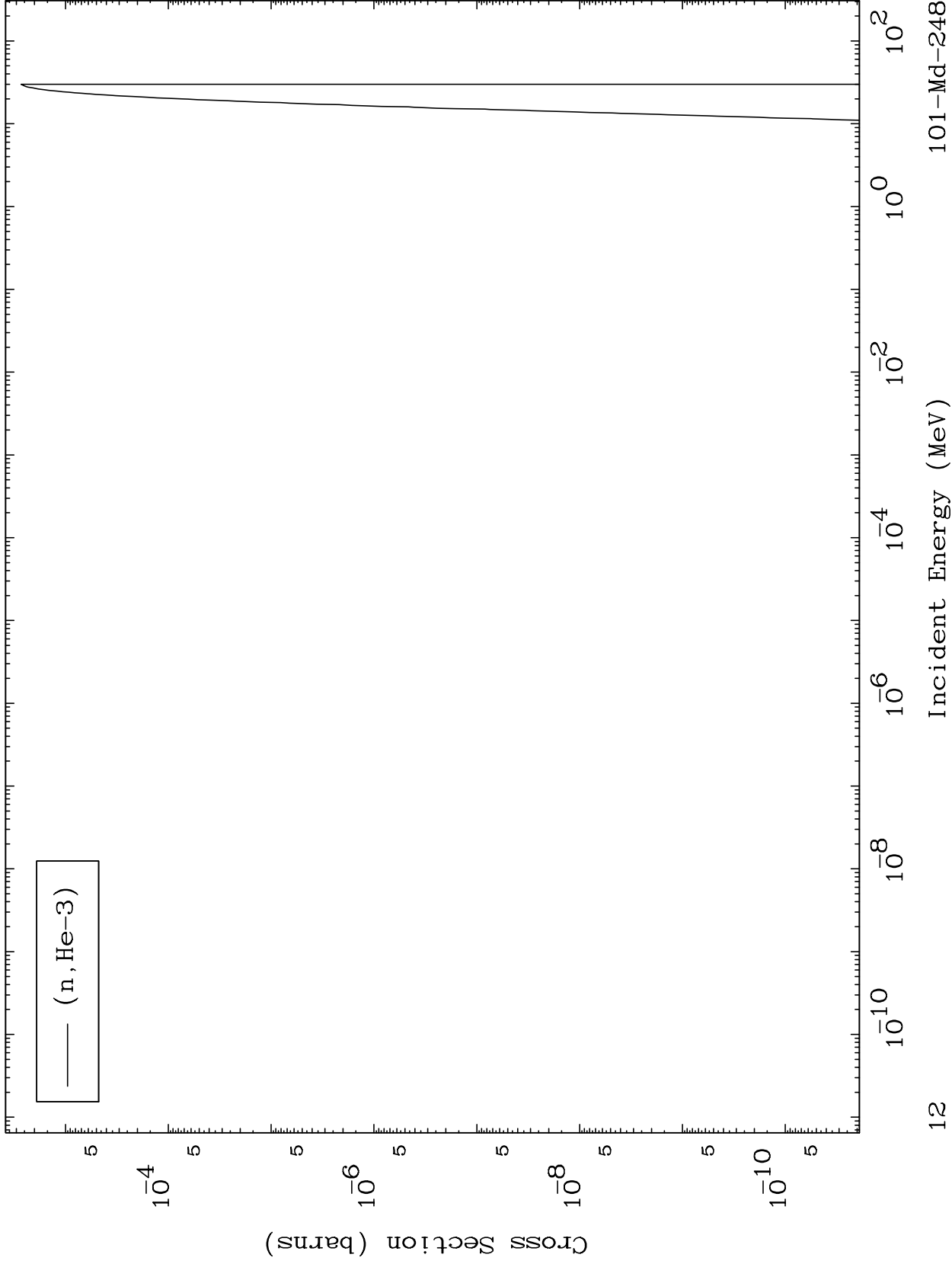
101-Md-248



MAT 9949

(n,He3) Levels
293 Kelvin Cross Sections

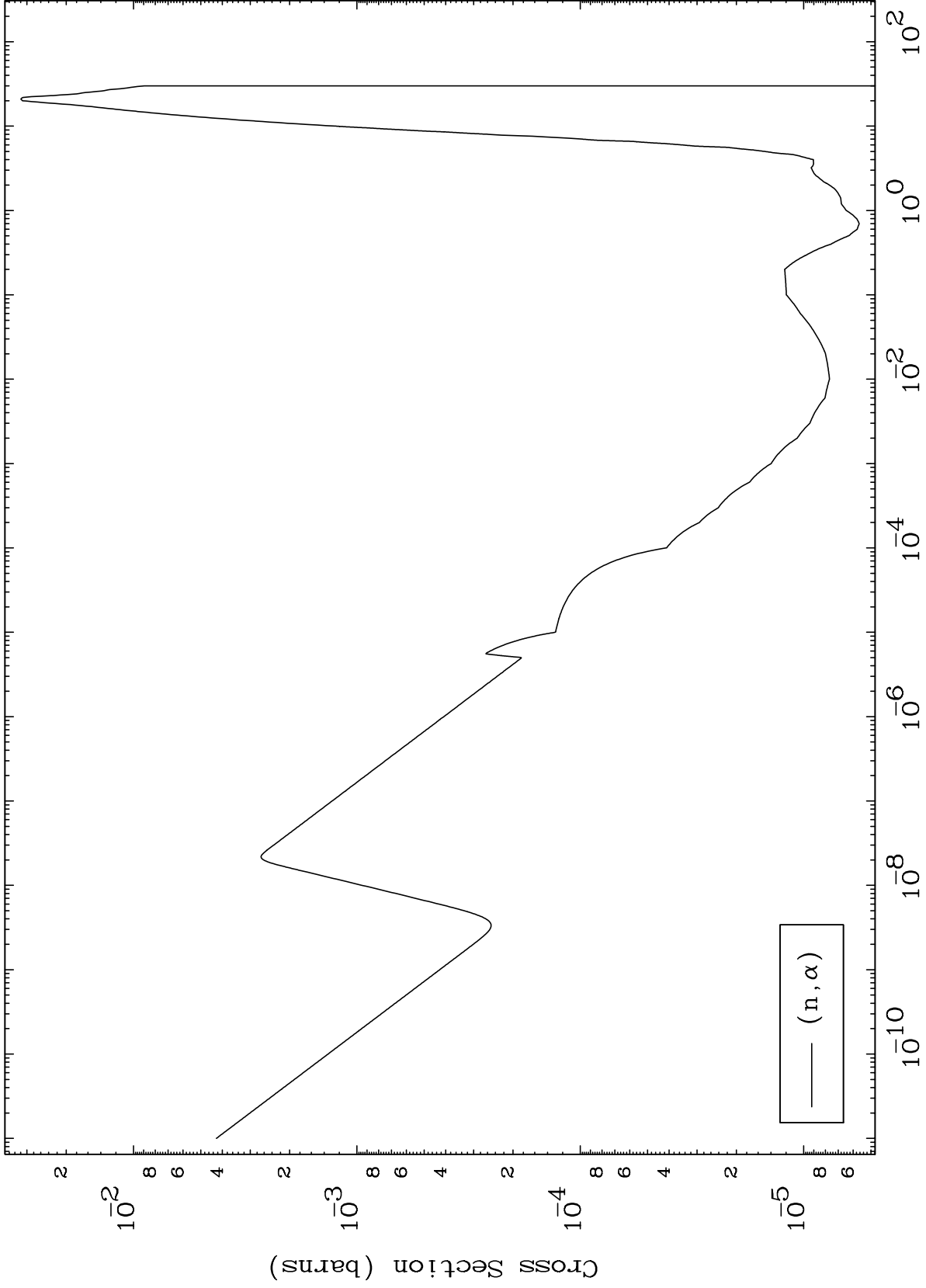
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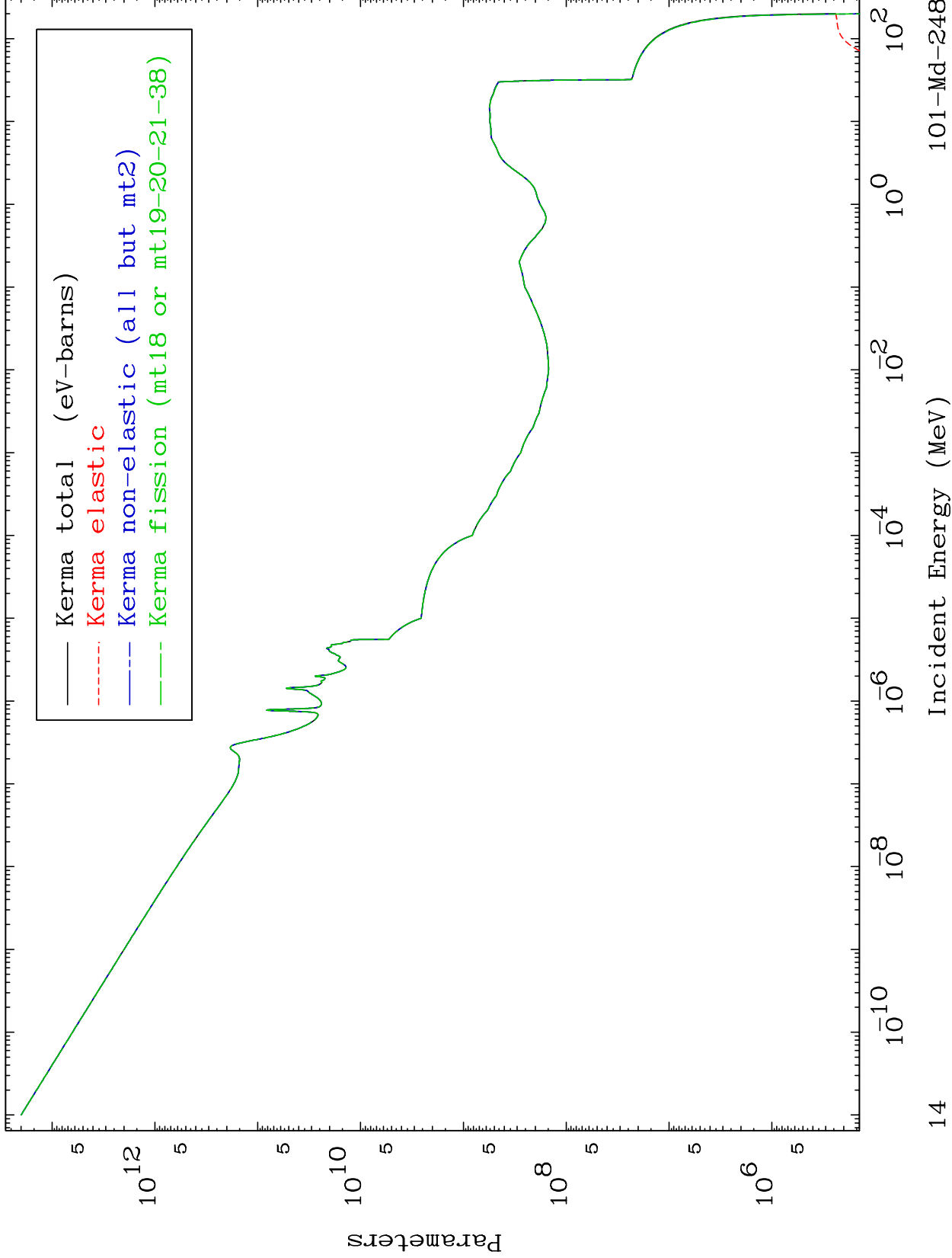


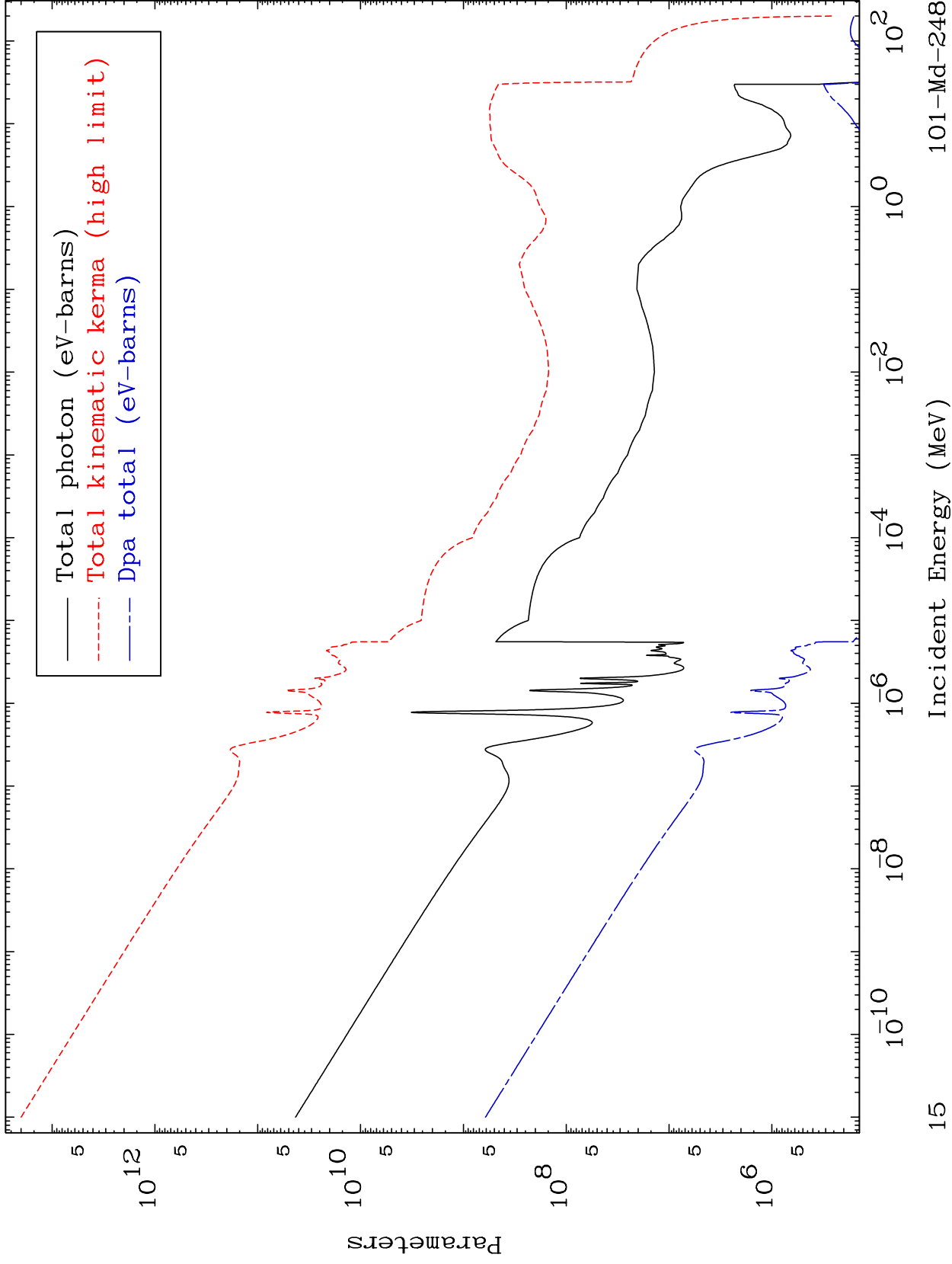
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(n, α) Levels
293 Kelvin Cross Sections

101-Md-248



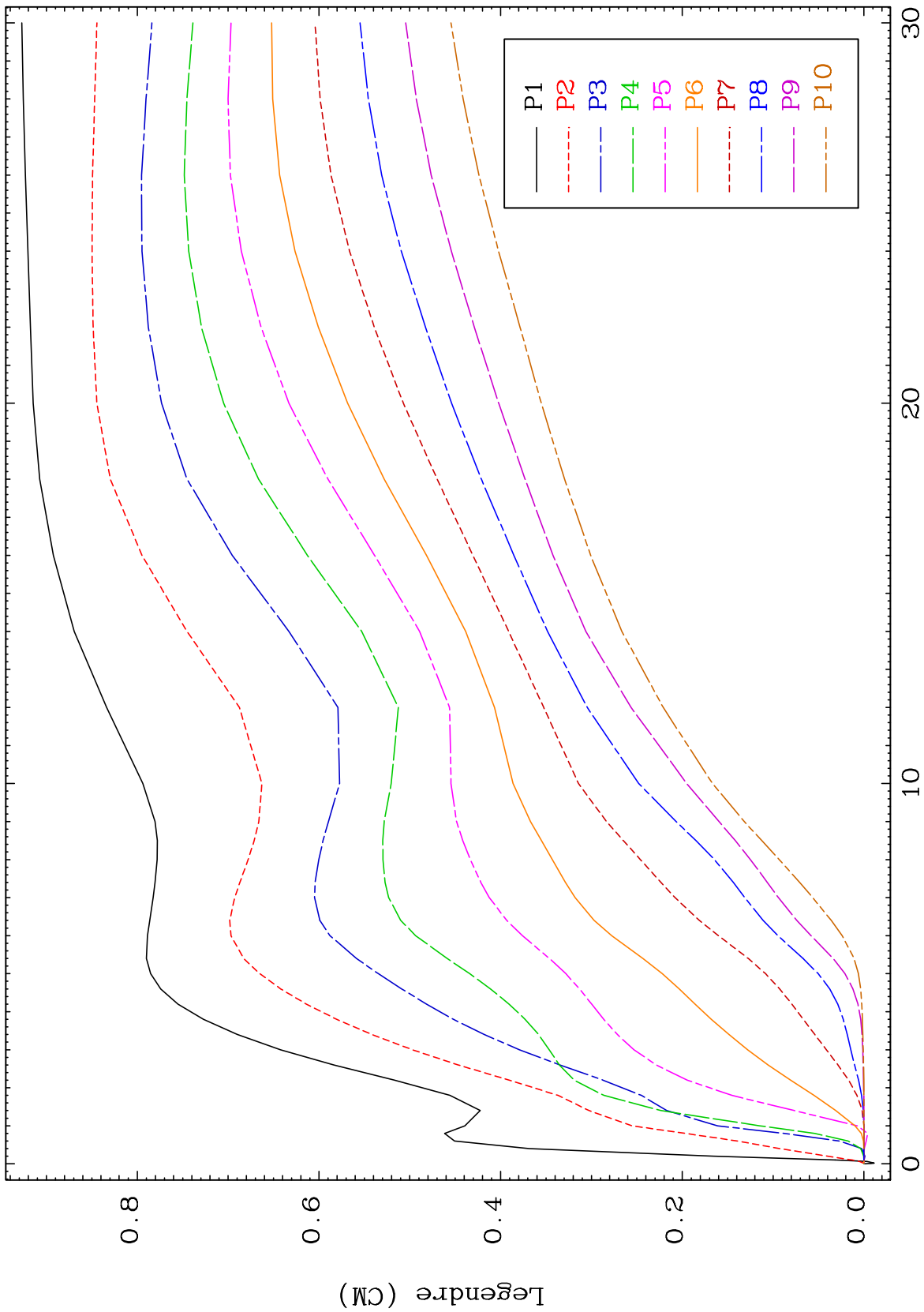




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101-Md-248

Elastic Legendre Coefficients



101-Md-248

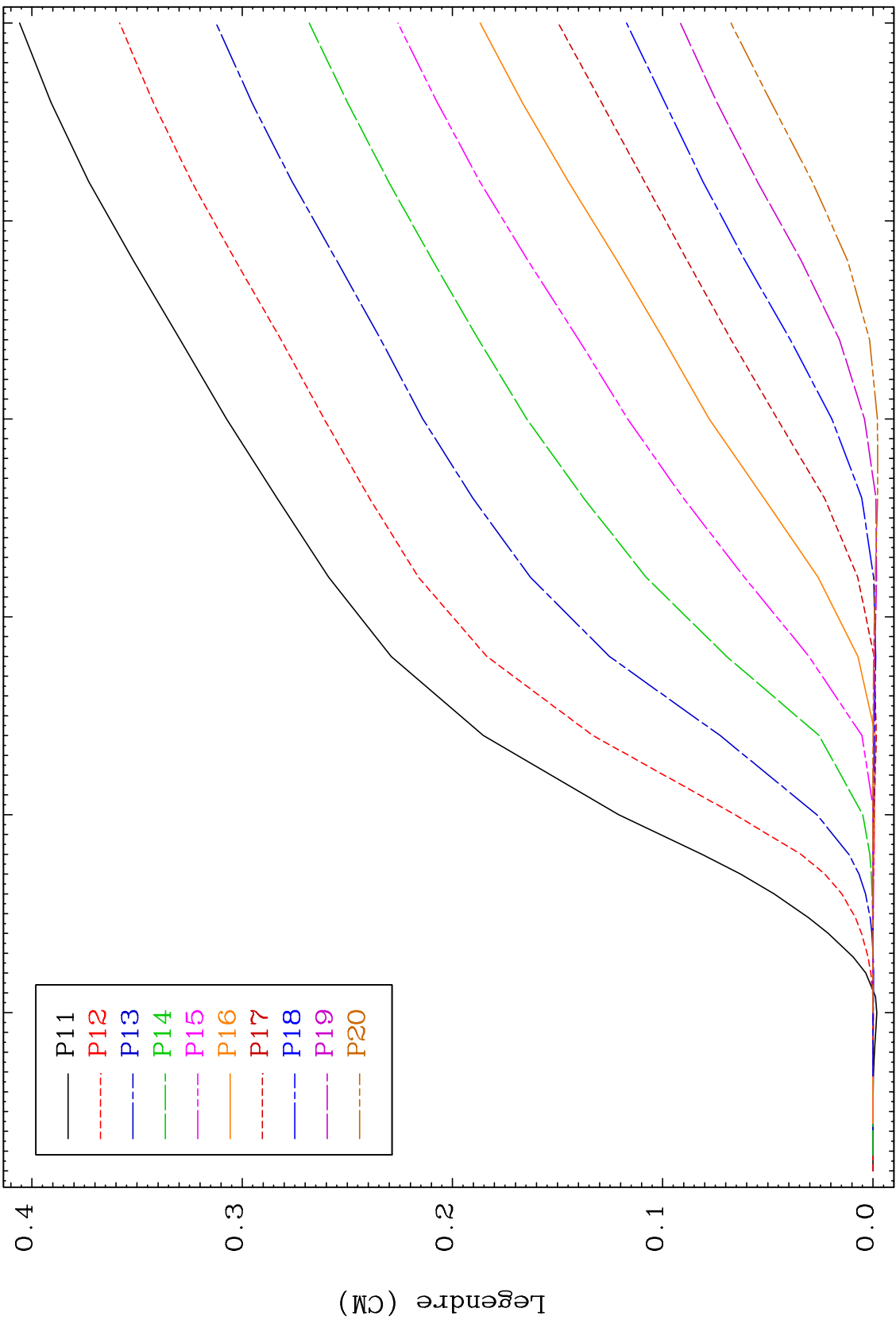
Incident Energy (MeV)

16

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Elastic Legendre Coefficients

101-Md-248



17

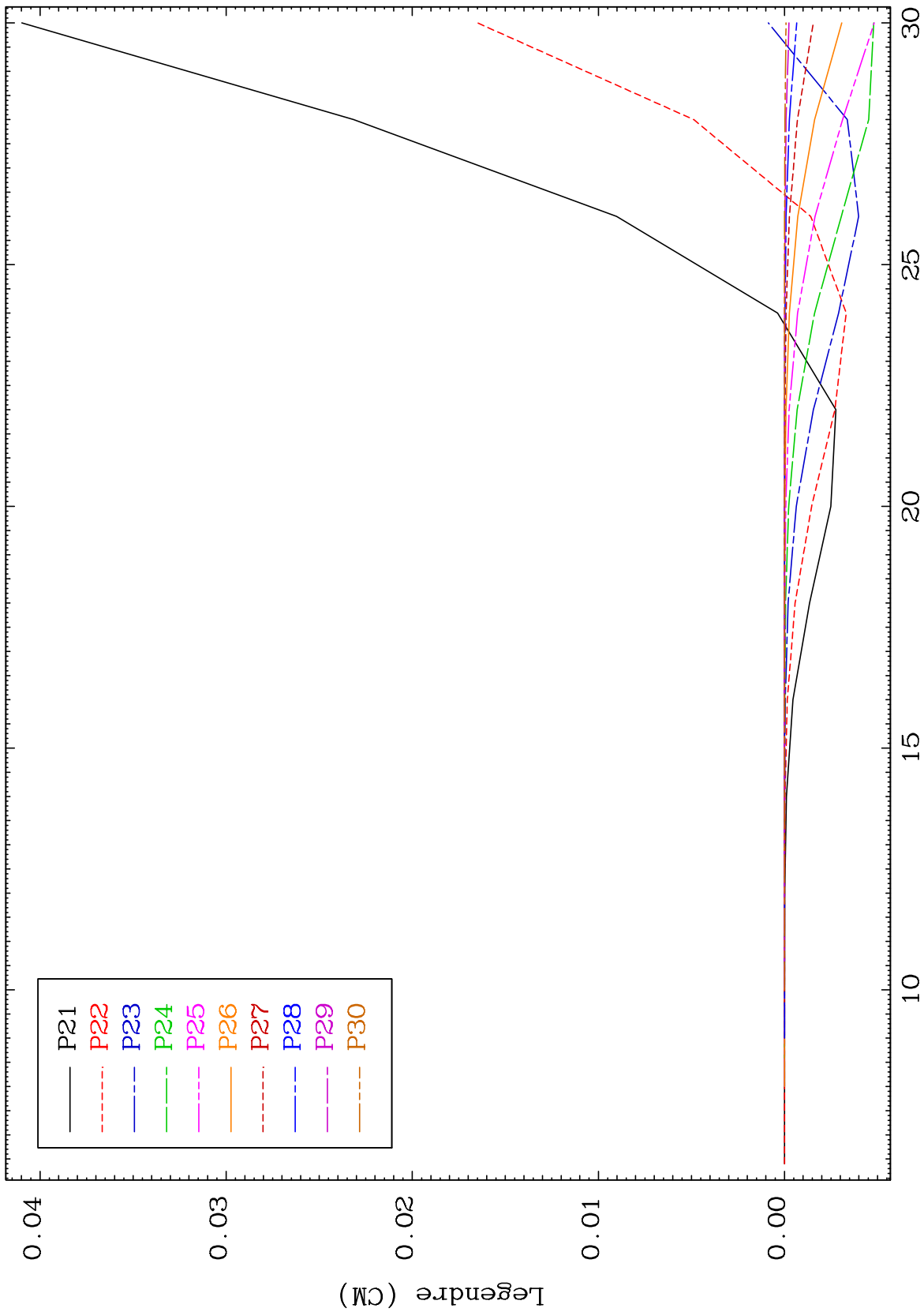
Incident Energy (MeV)

101-Md-248

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Elastic Legendre Coefficients

101-Md-248



18

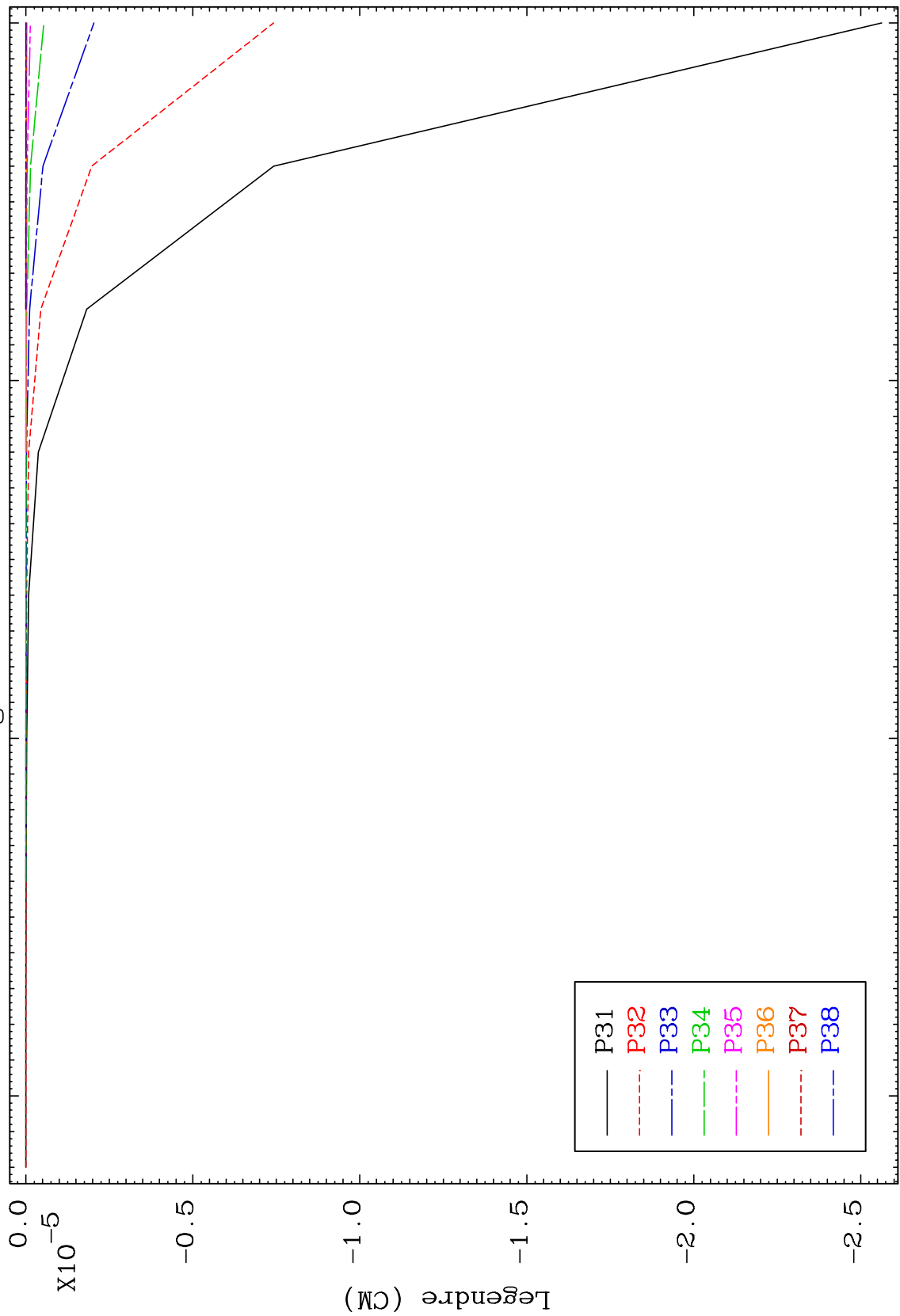
Incident Energy (MeV)

101-Md-248

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101-Md-248

Elastic Legendre Coefficients



19

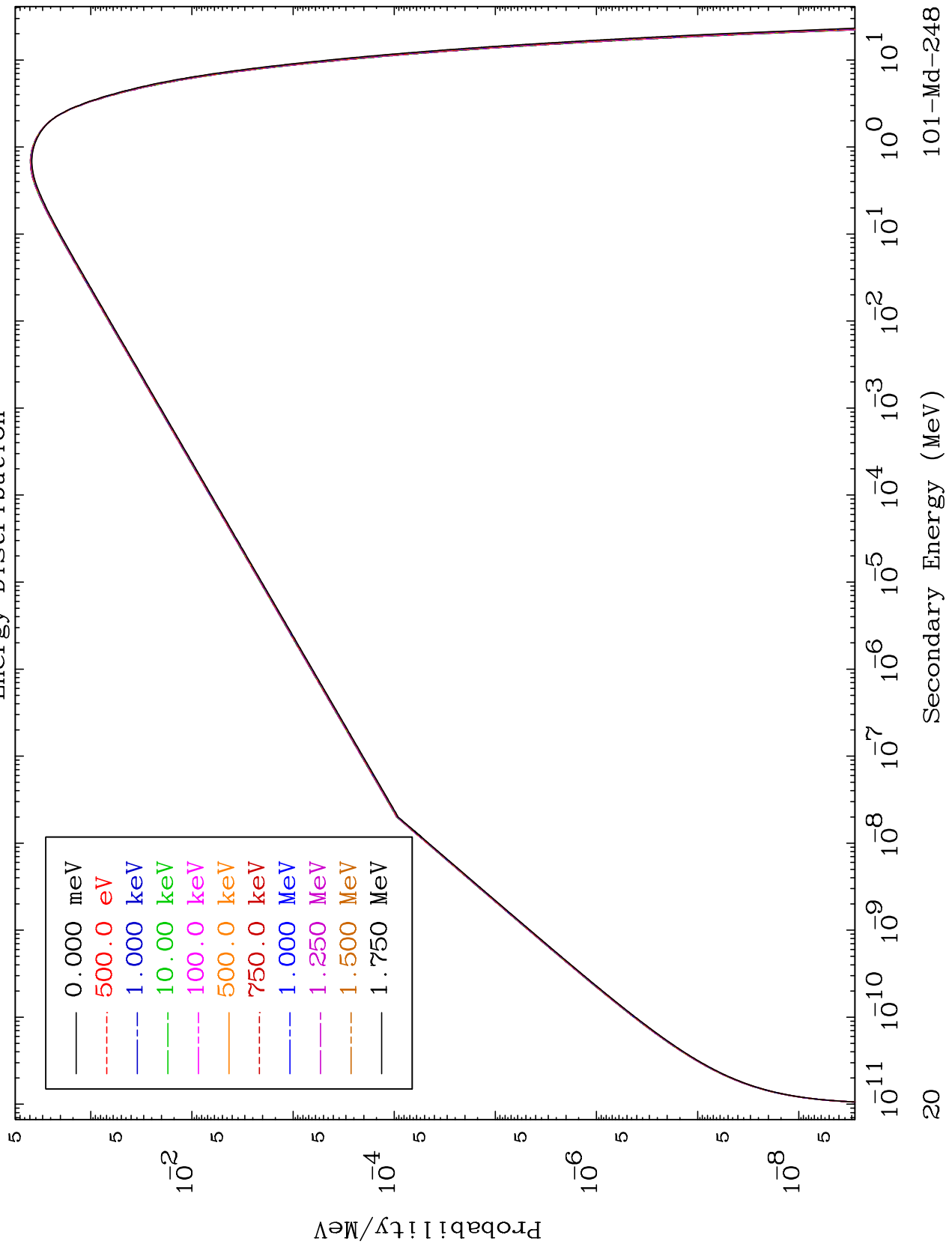
Incident Energy (MeV)

101-Md-248

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Fission Energy Distribution

101-Md-248



101-Md-248

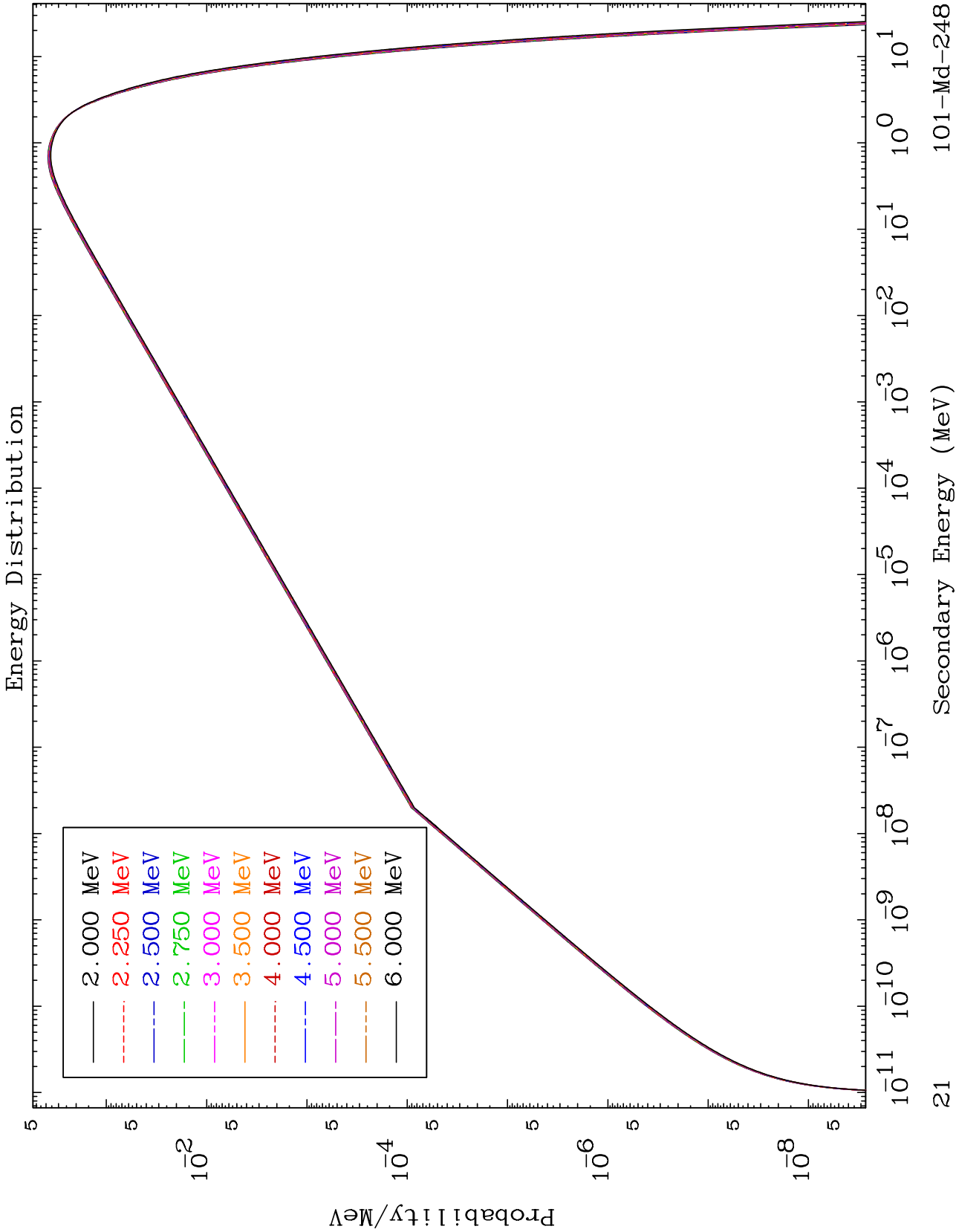
Secondary Energy (MeV)

20

MAT 9949

Fission

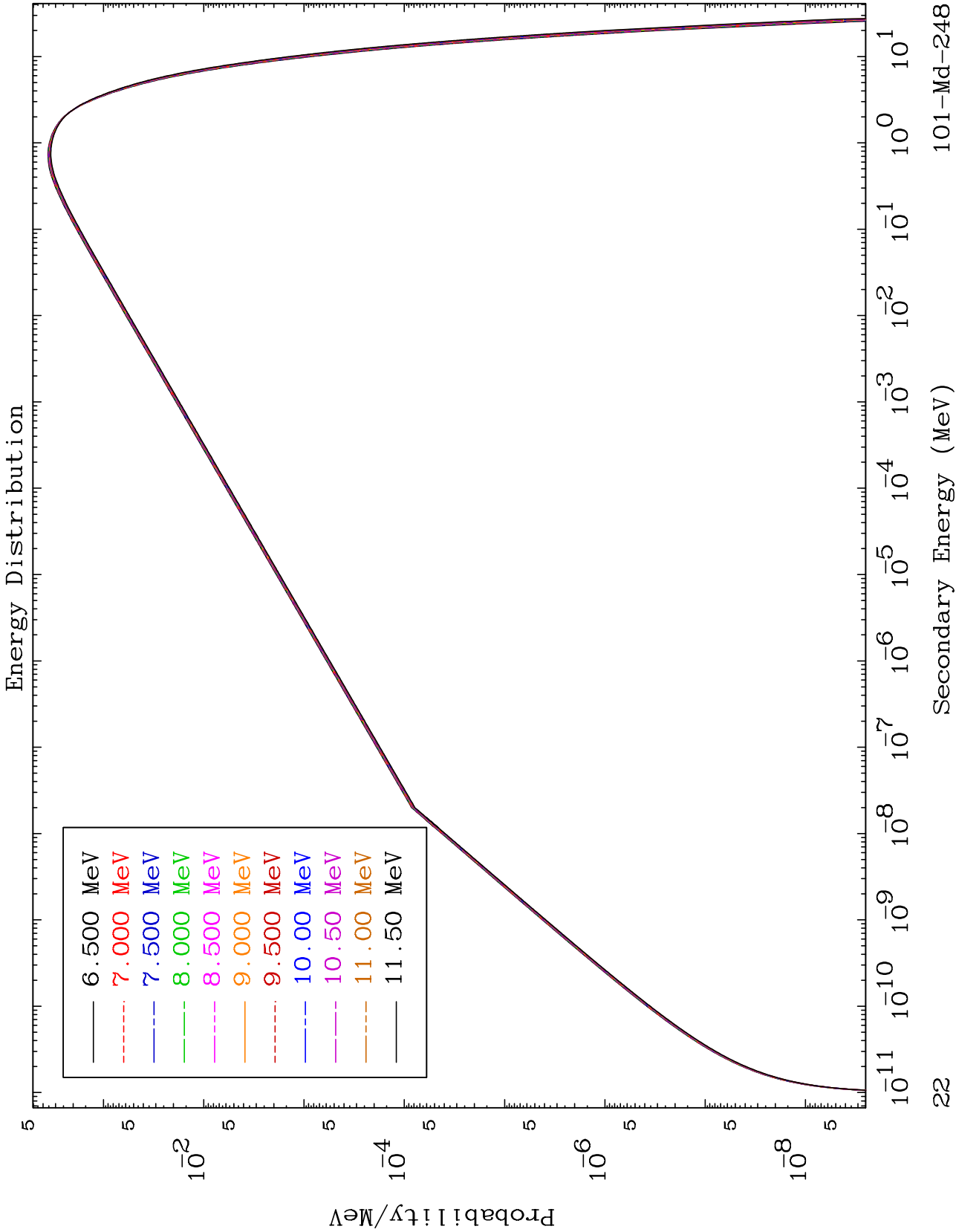
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MAT 9949

Fission

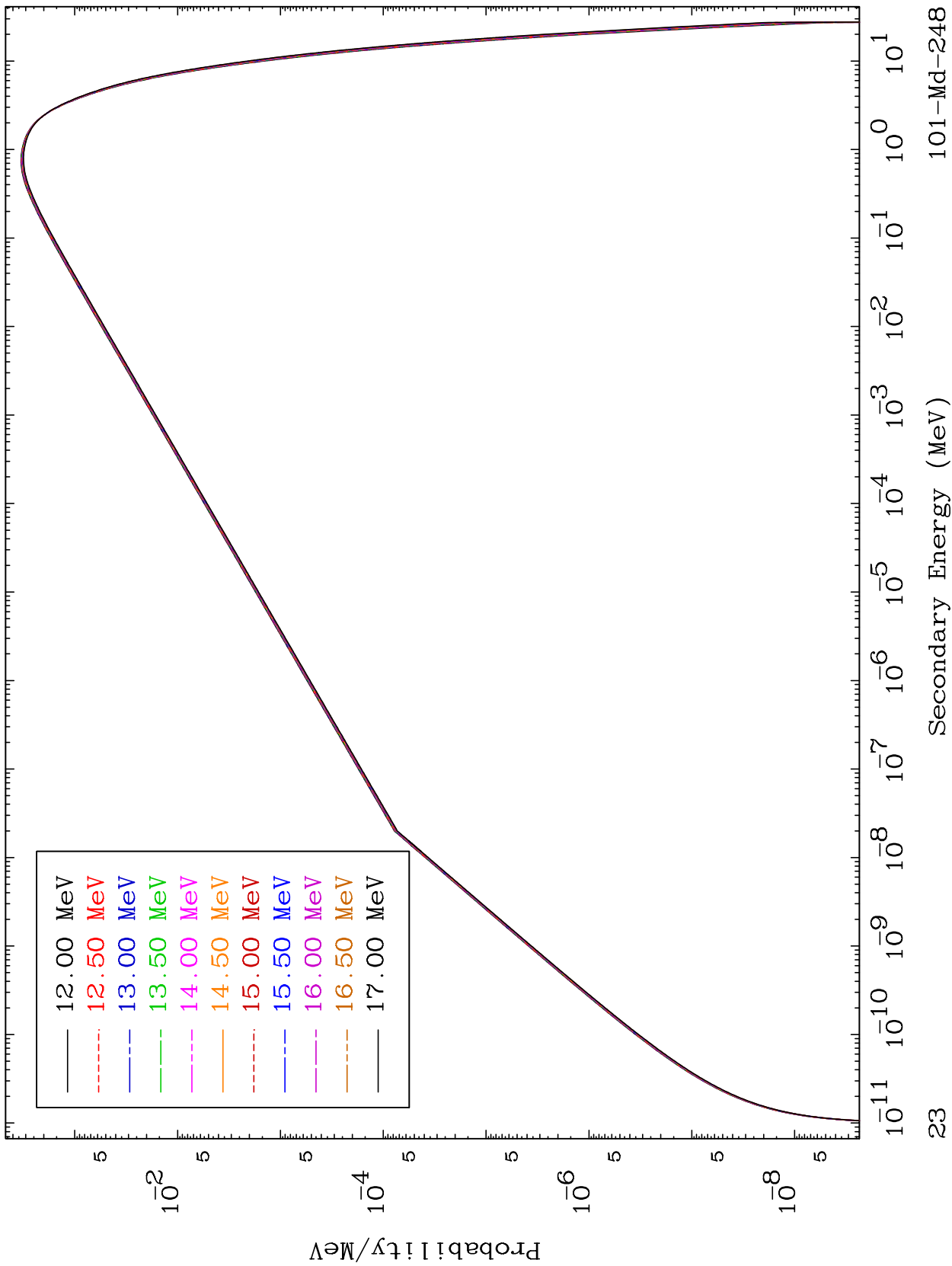
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Fission Energy Distribution

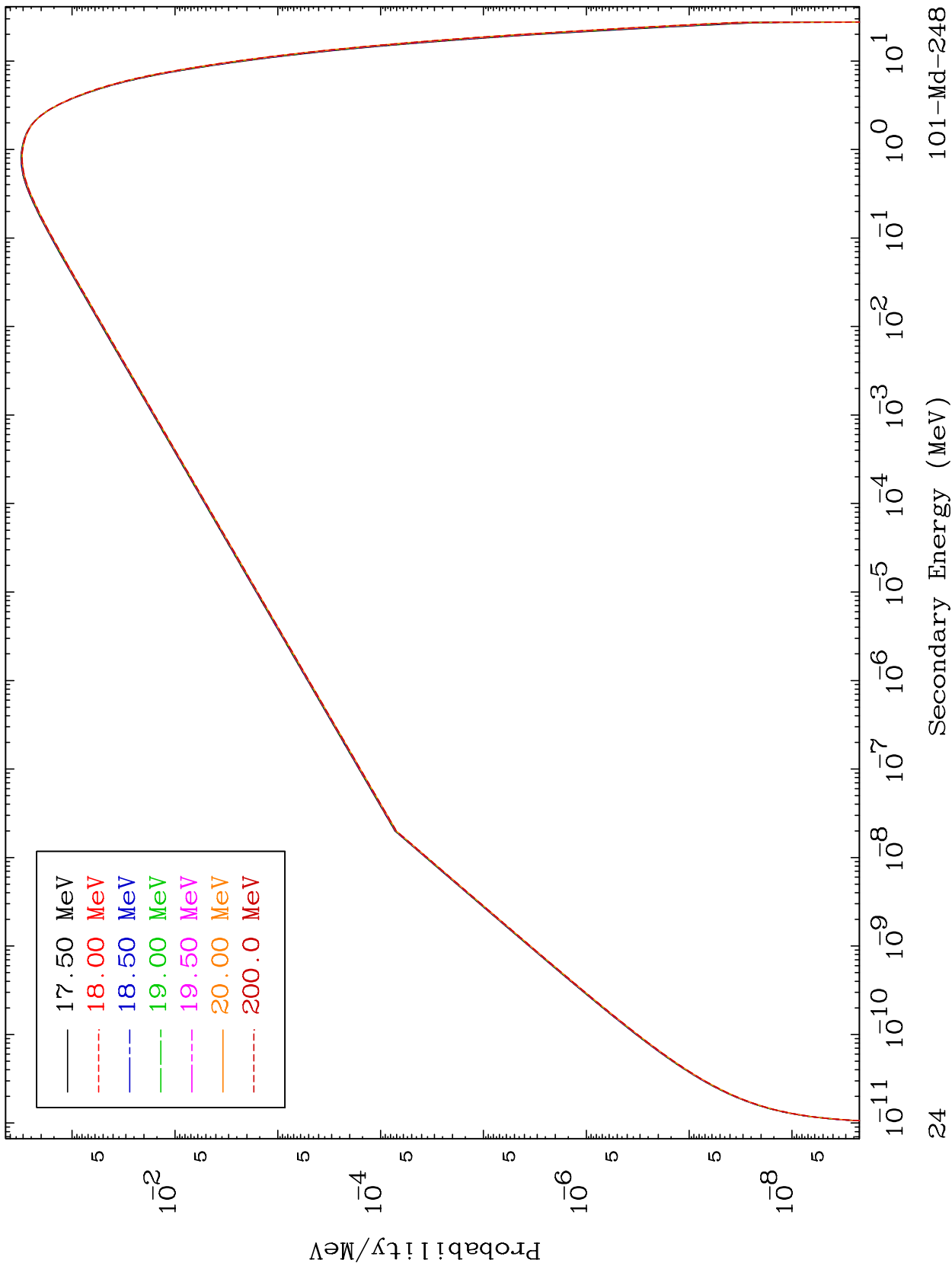
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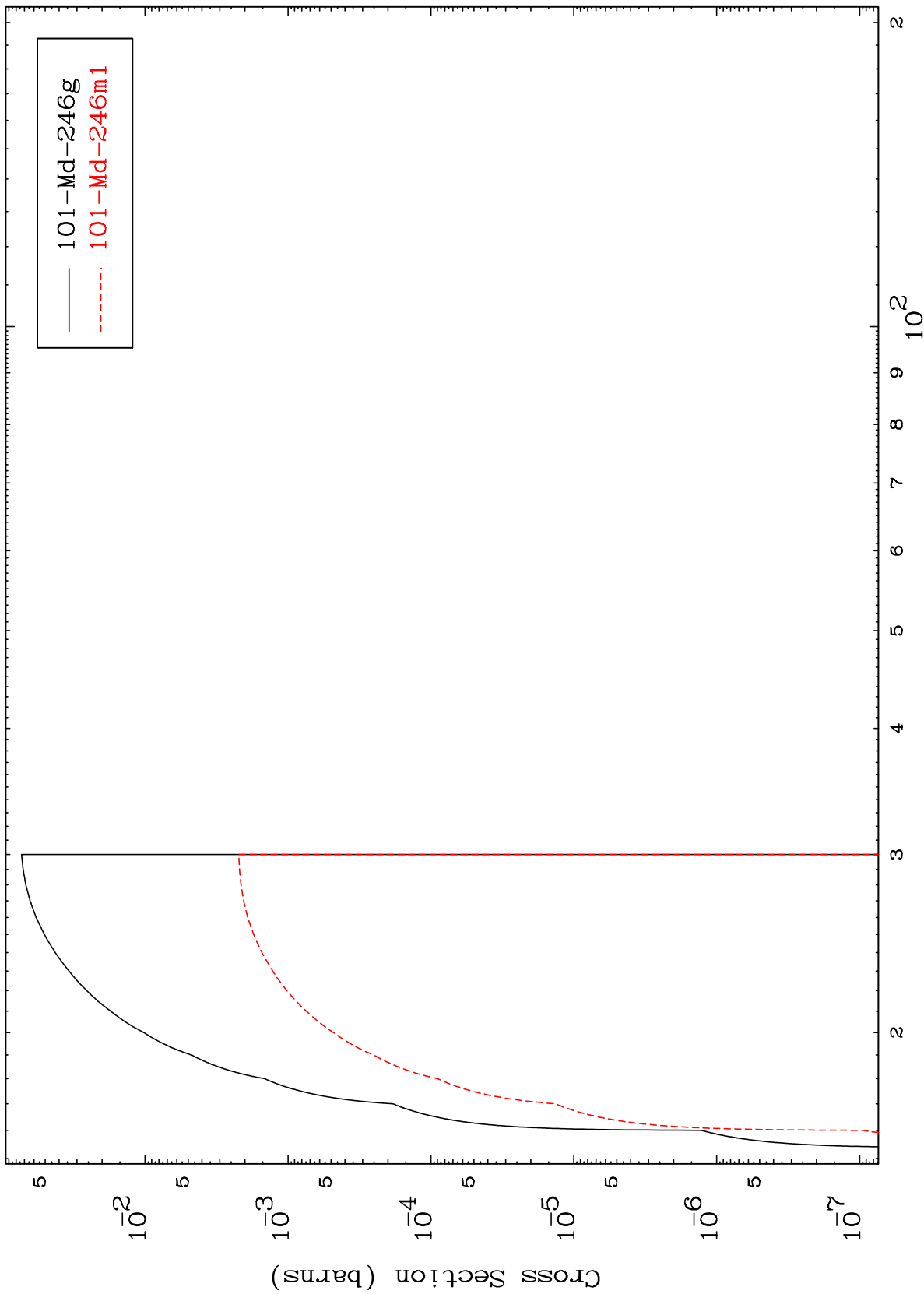
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Fission Energy Distribution

101-Md-248



(n,3n)
Radionuclide Production Cross Section

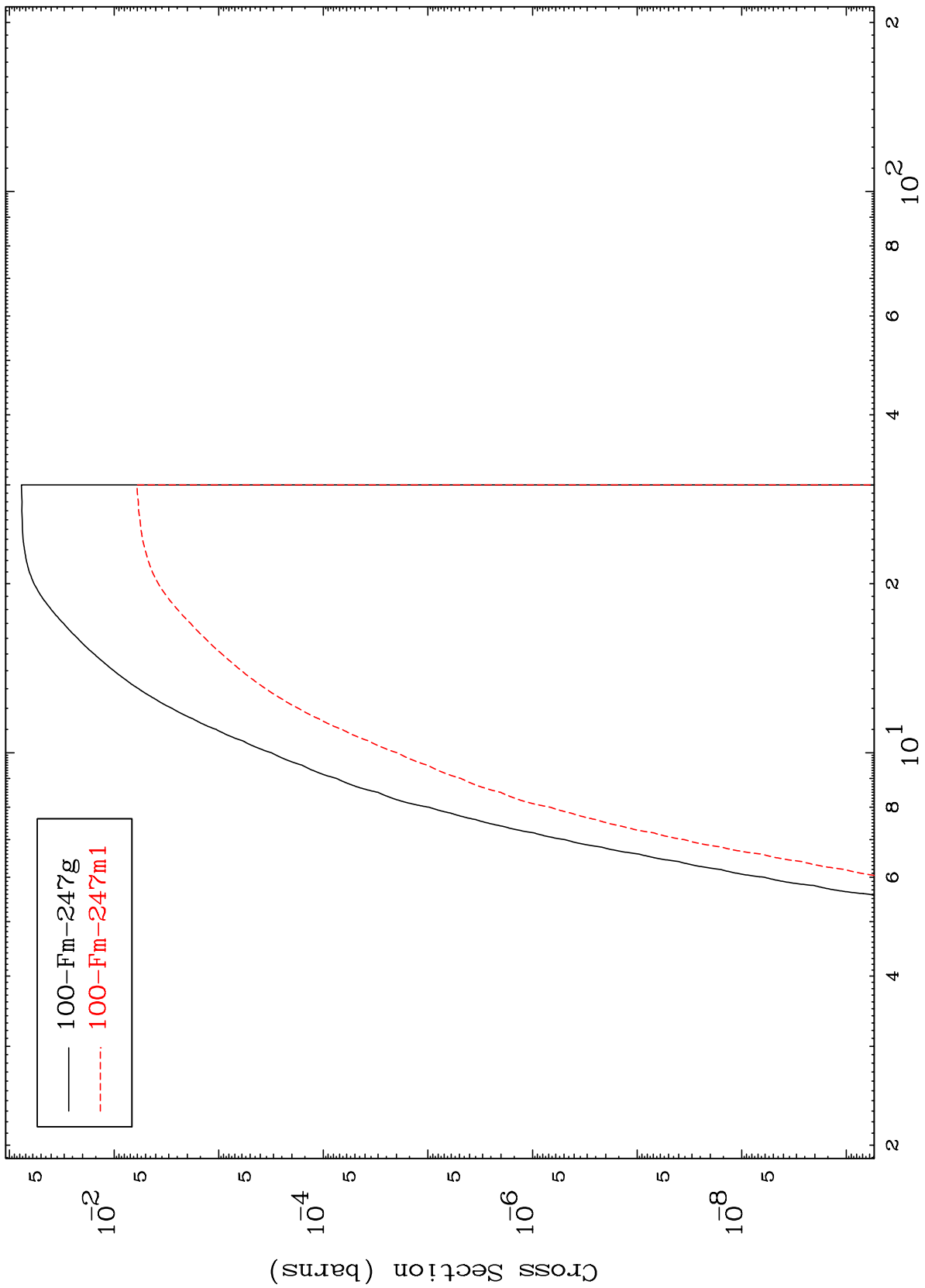


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(n,n') p

101-Md-248

Radionuclide Production Cross Section



26

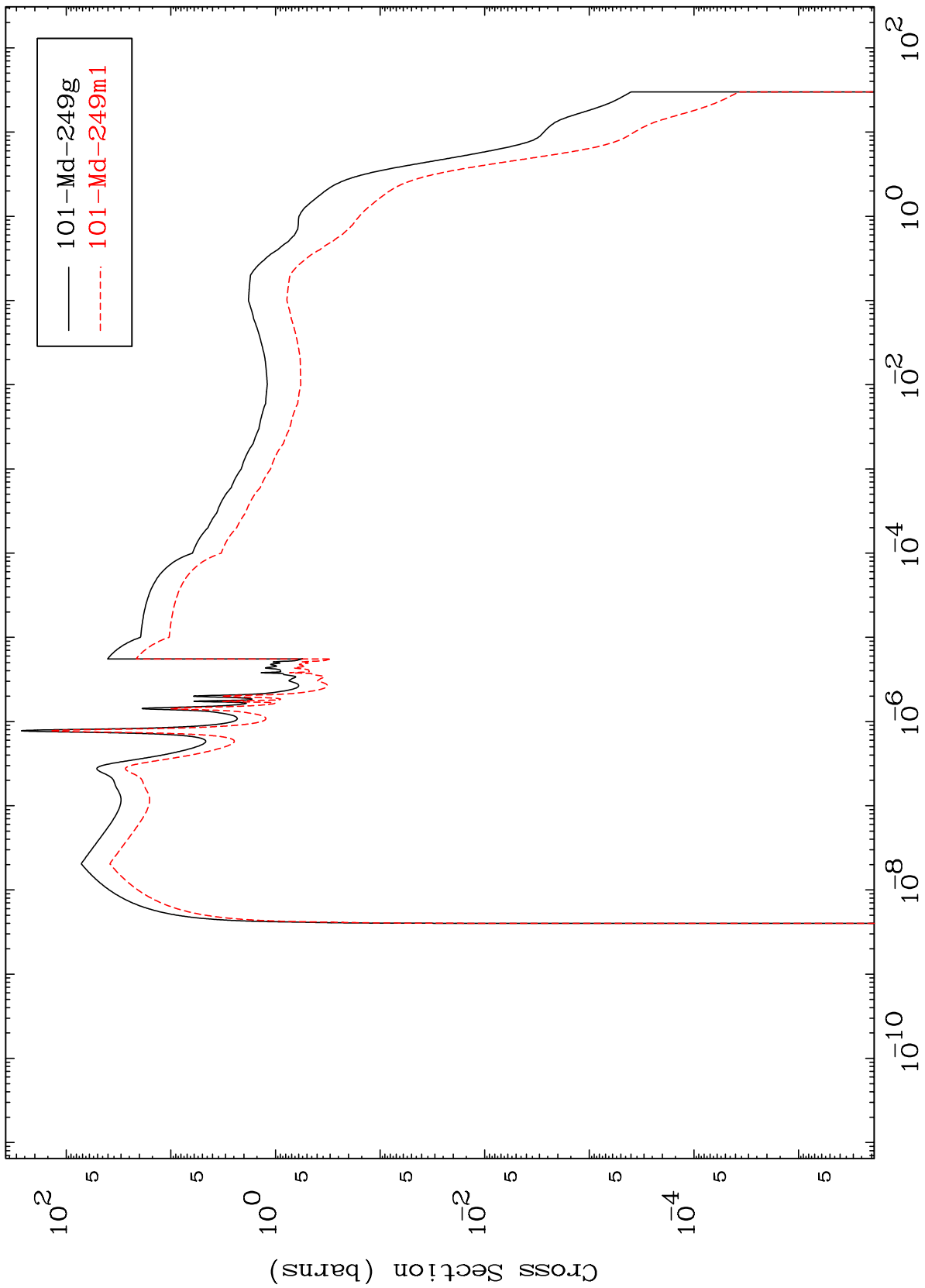
Incident Energy (MeV)

101-Md-248

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101-Md-248

(n, γ)
Radionuclide Production Cross Section



27

101-Md-248

MAT 9949

(n,d)

101-Md-248

Radionuclide Production Cross Section

