

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

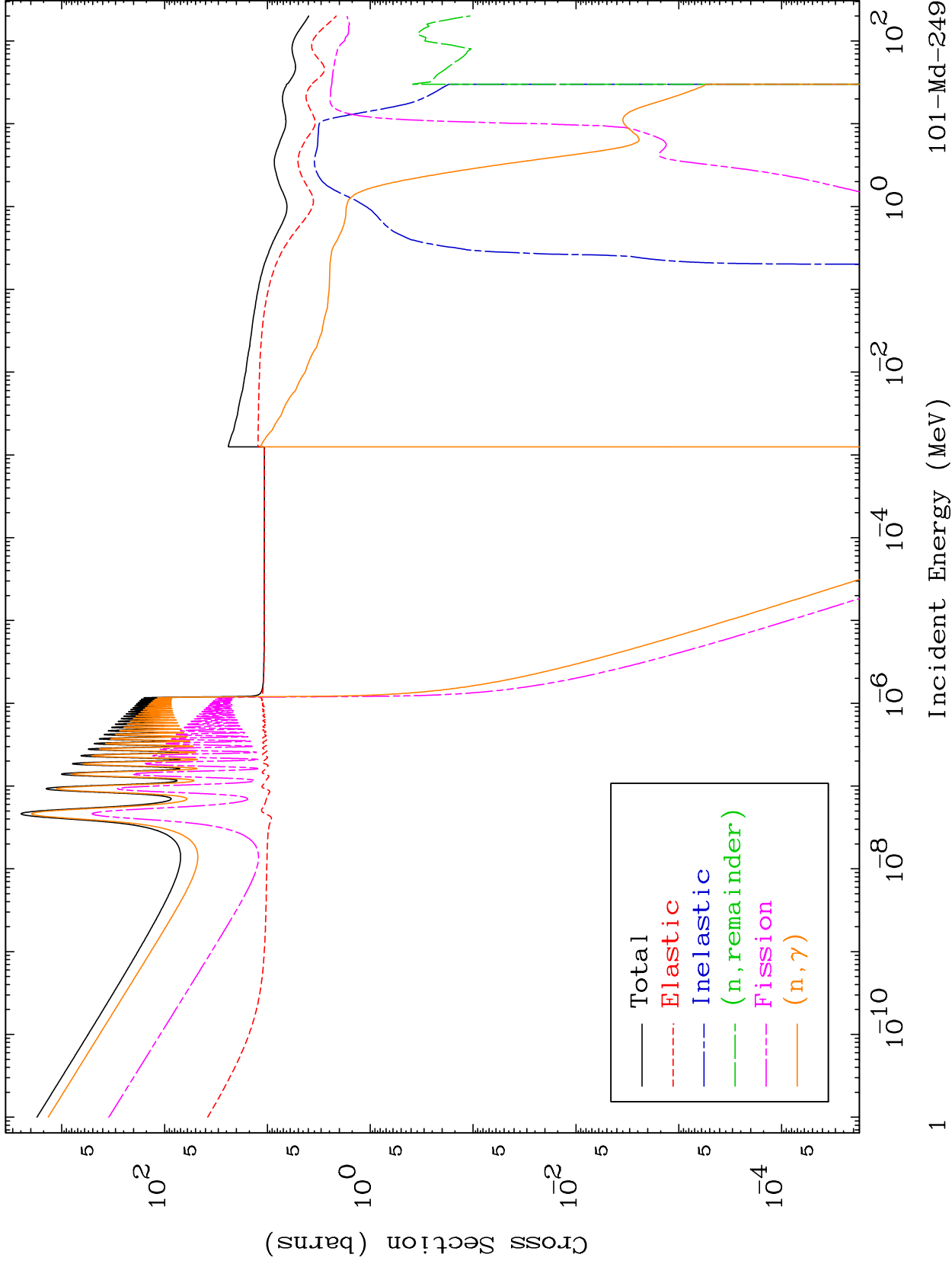
Press Mouse Button to Start

MAT 9950

Major

293 Kelvin Cross Sections

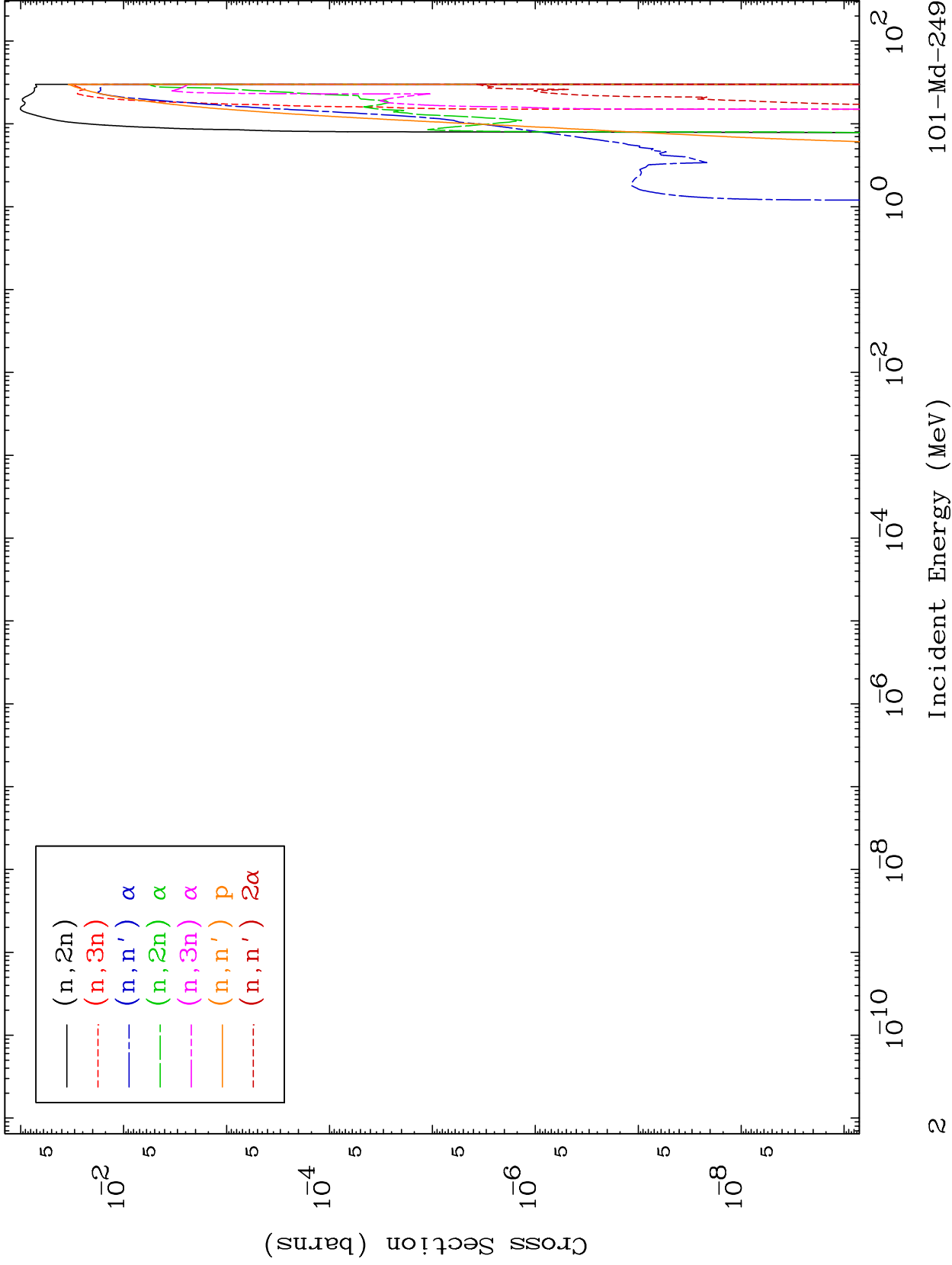
101-Md-249



MAT 9950

Neutron Production
293 Kelvin Cross Sections

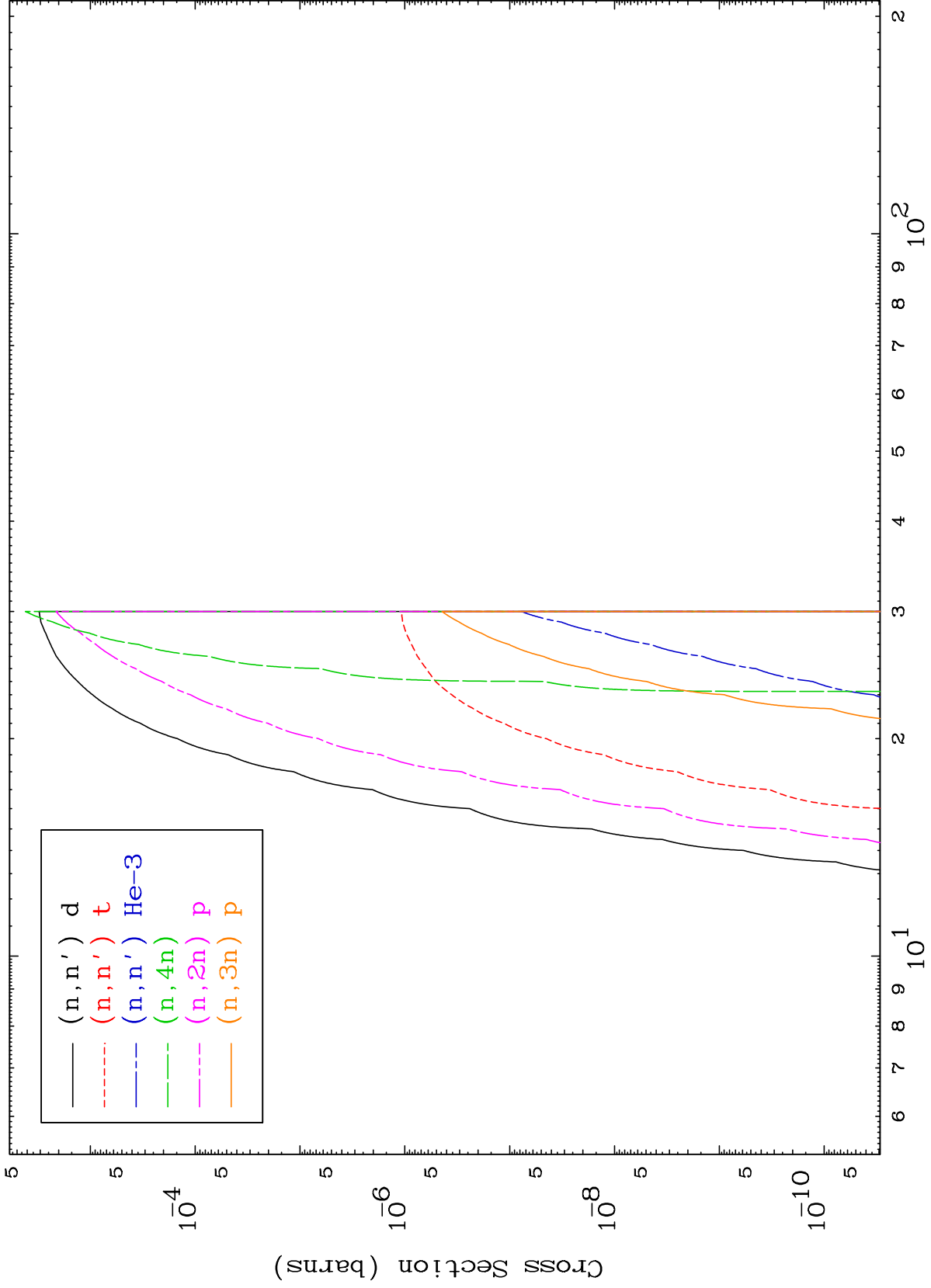
101-Md-249



MAT 9950

Neutron Production
293 Kelvin Cross Sections

101-Md-249



3

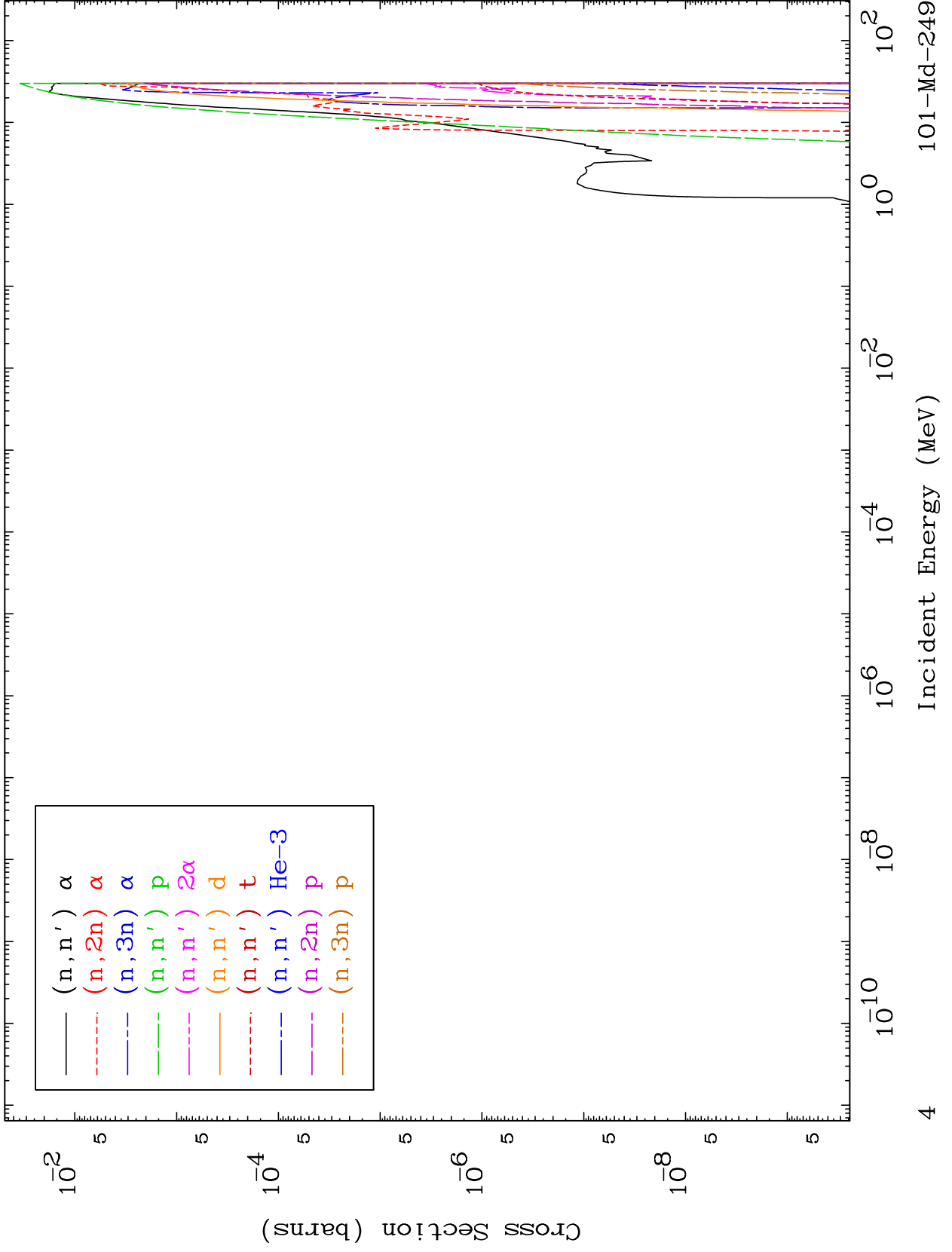
Incident Energy (MeV)

101-Md-249

MAT 9950

Charged Particle
293 Kelvin Cross Sections

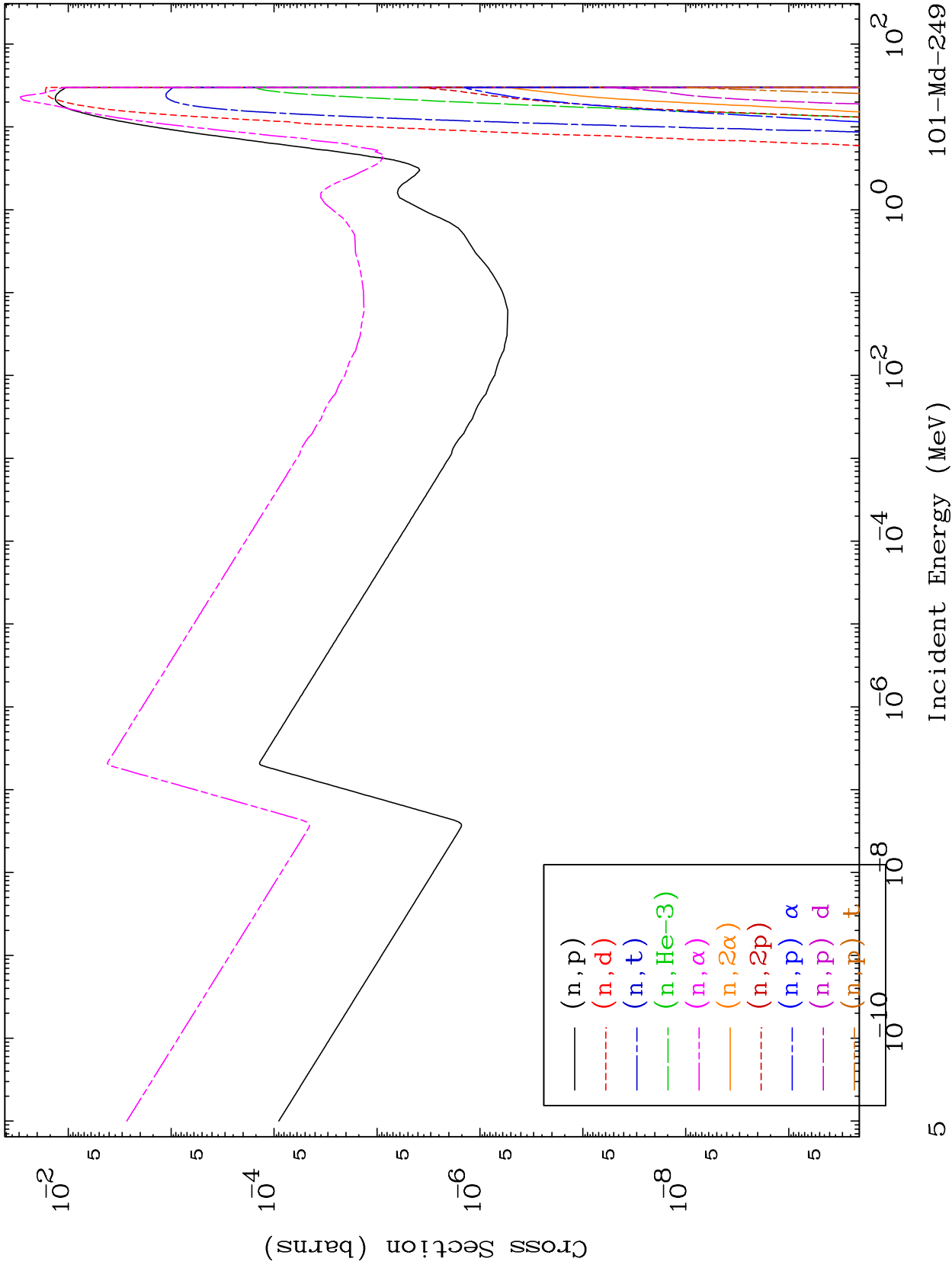
101-Md-249



MAT 9950

Charged Particle
293 Kelvin Cross Sections

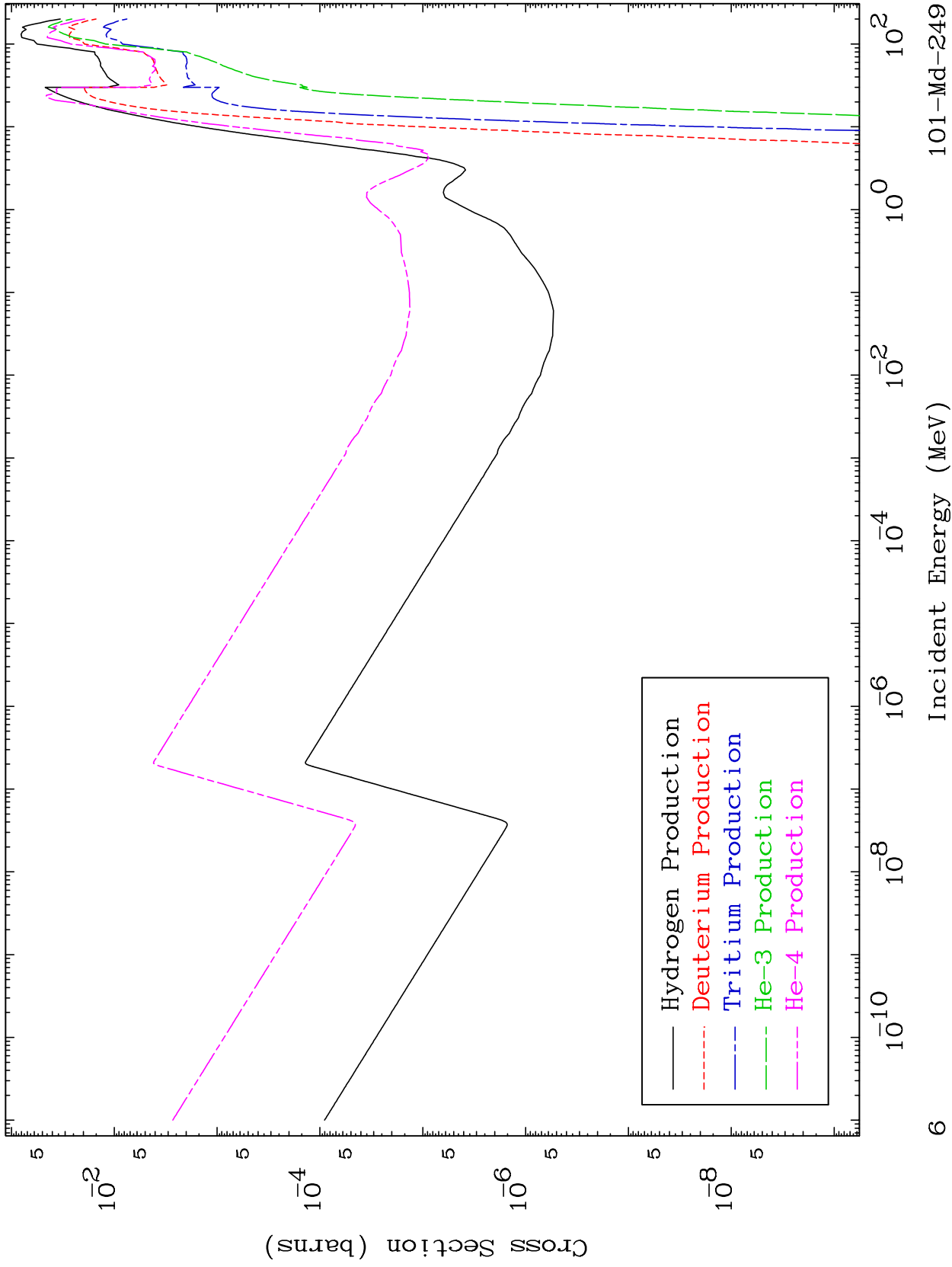
101-Md-249



MAT 9950

Particle Production
293 Kelvin Cross Sections

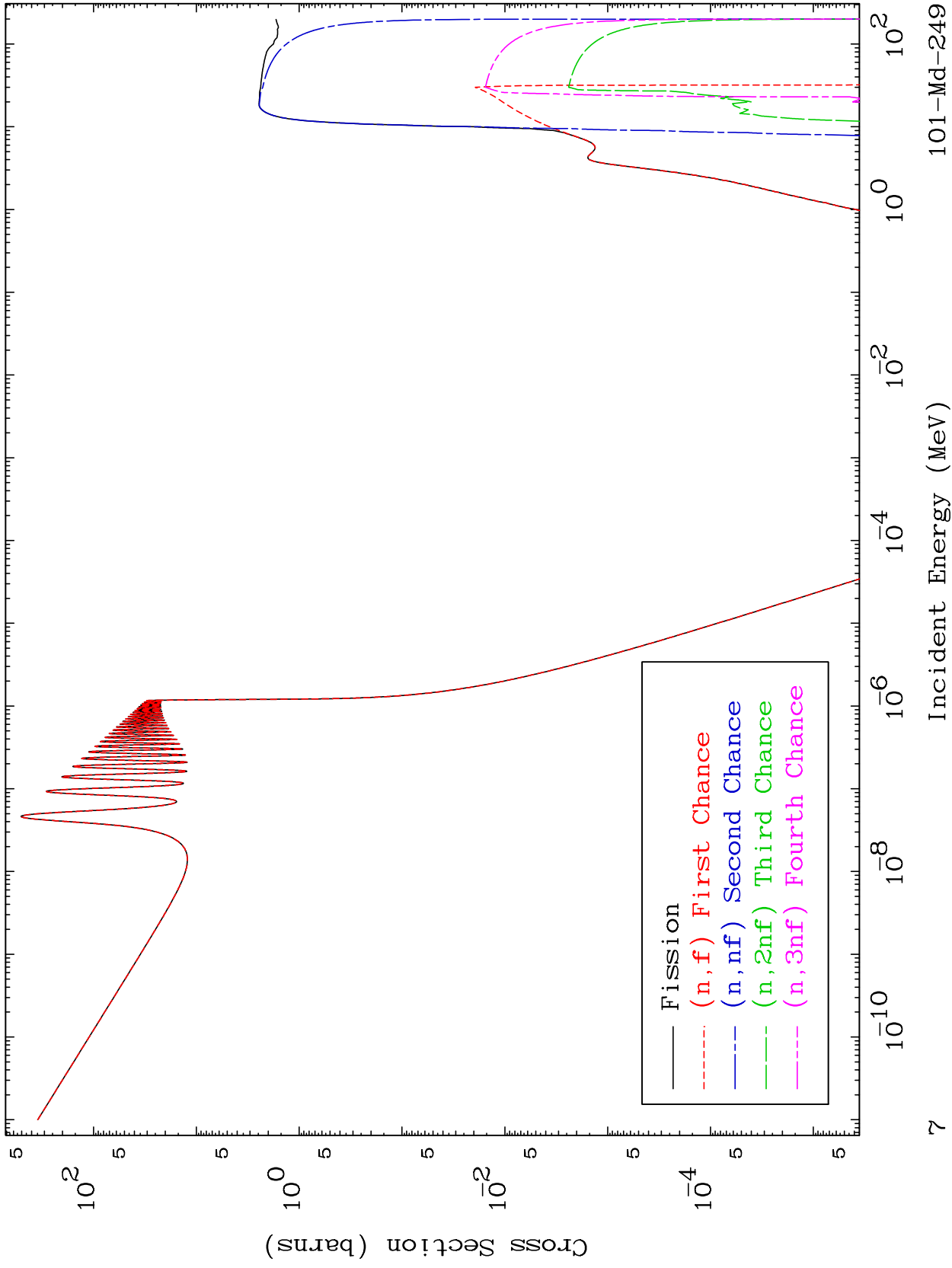
101-Md-249



MAT 9950

Fission
293 Kelvin Cross Sections

101-Md-249

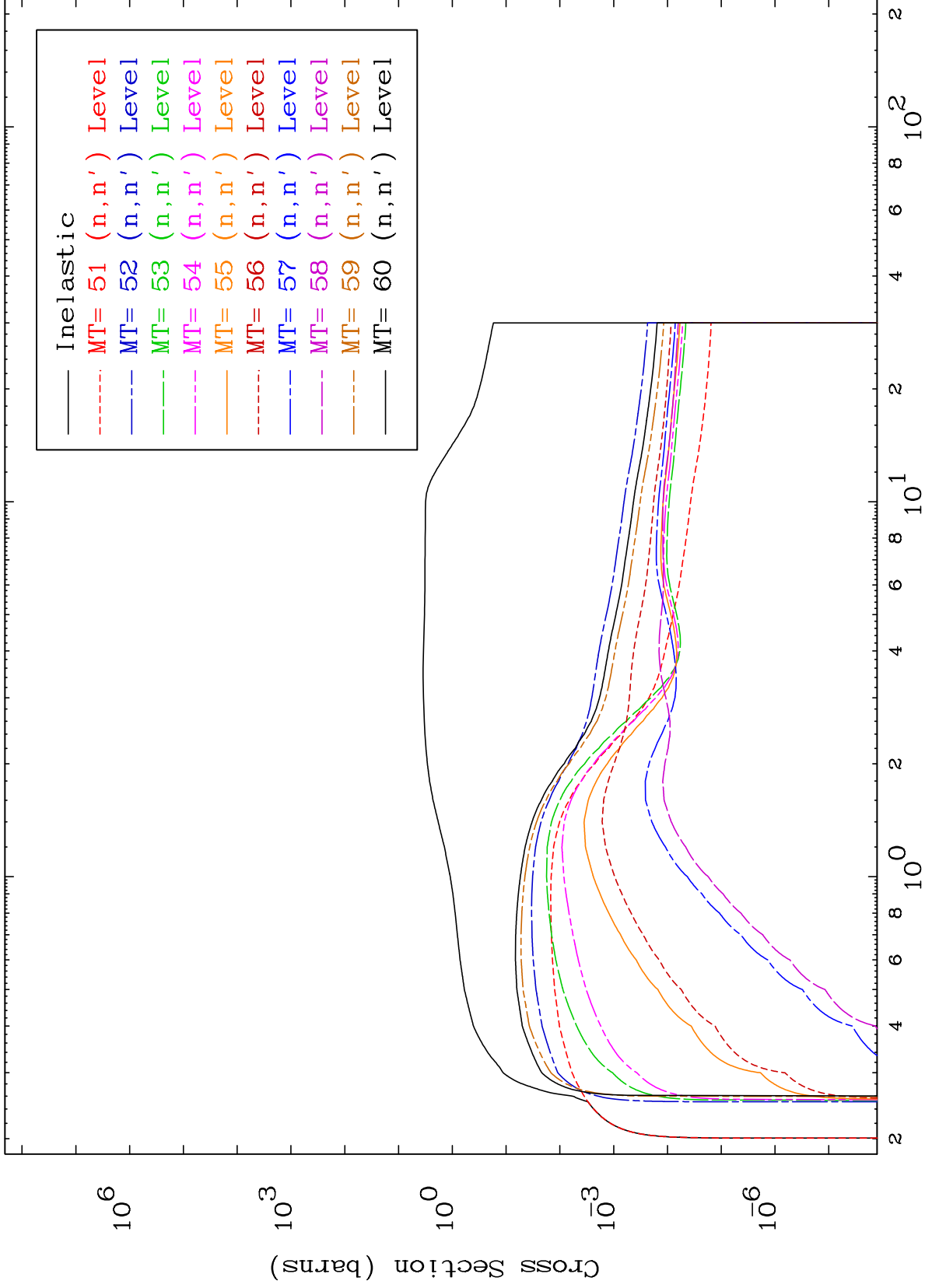


MAT 9950

(n,n') Level

101-Md-249

293 Kelvin Cross Sections



8

Incident Energy (MeV)

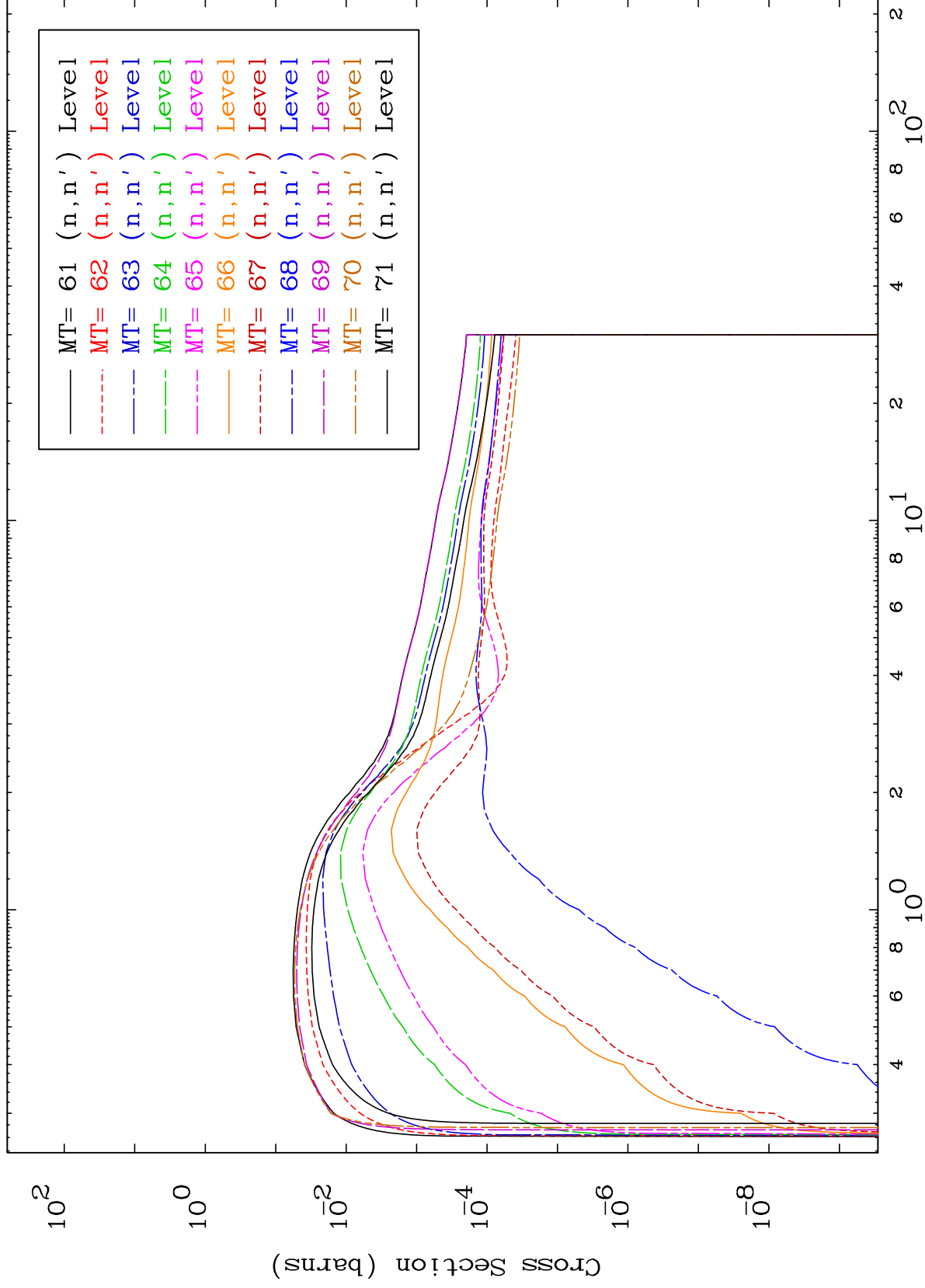
101-Md-249

MAT 9950

(n,n') Level

293 Kelvin Cross Sections

101-Md-249

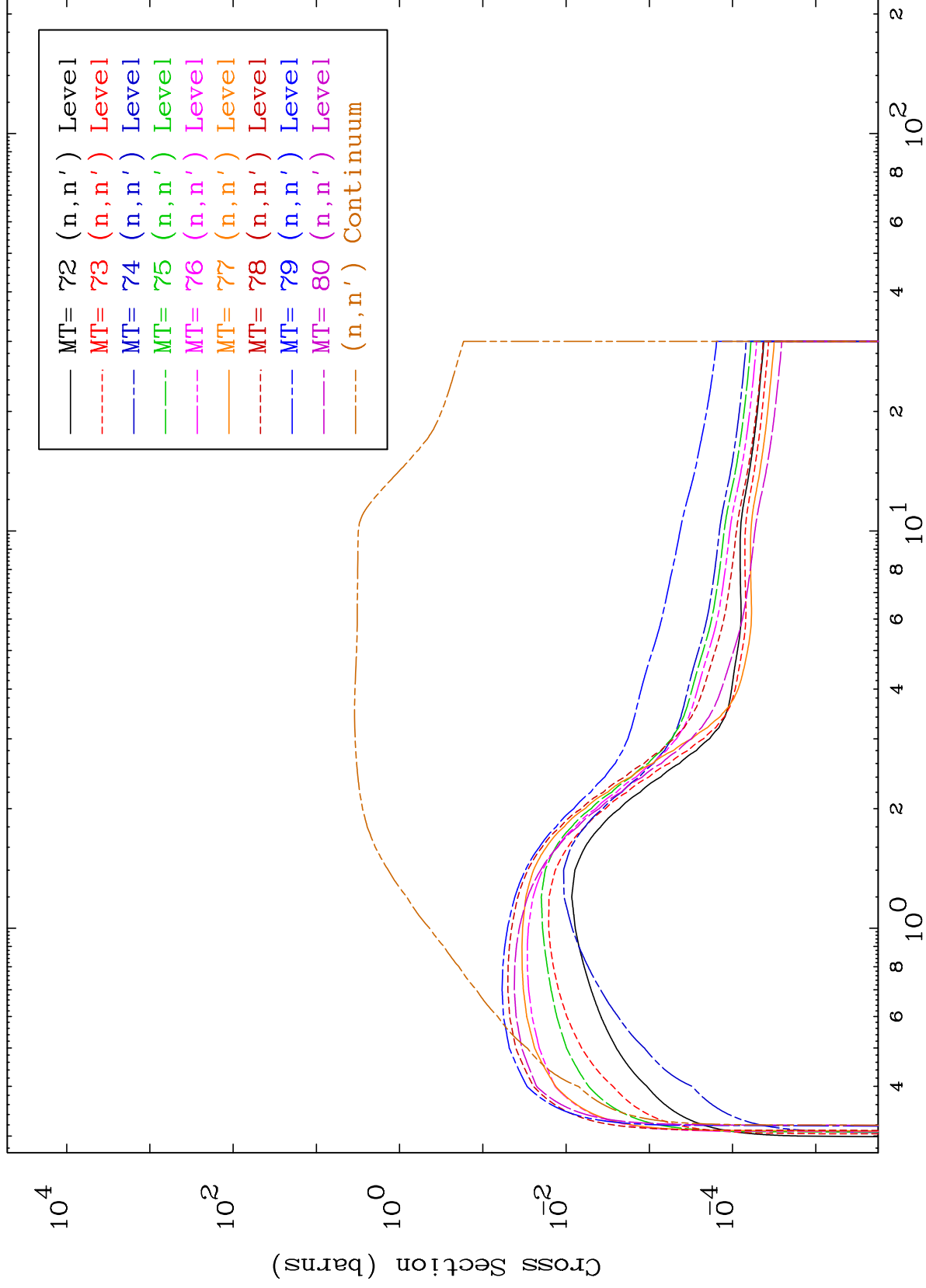


MAT 9950

(n,n') Level

293 Kelvin Cross Sections

101-Md-249



10

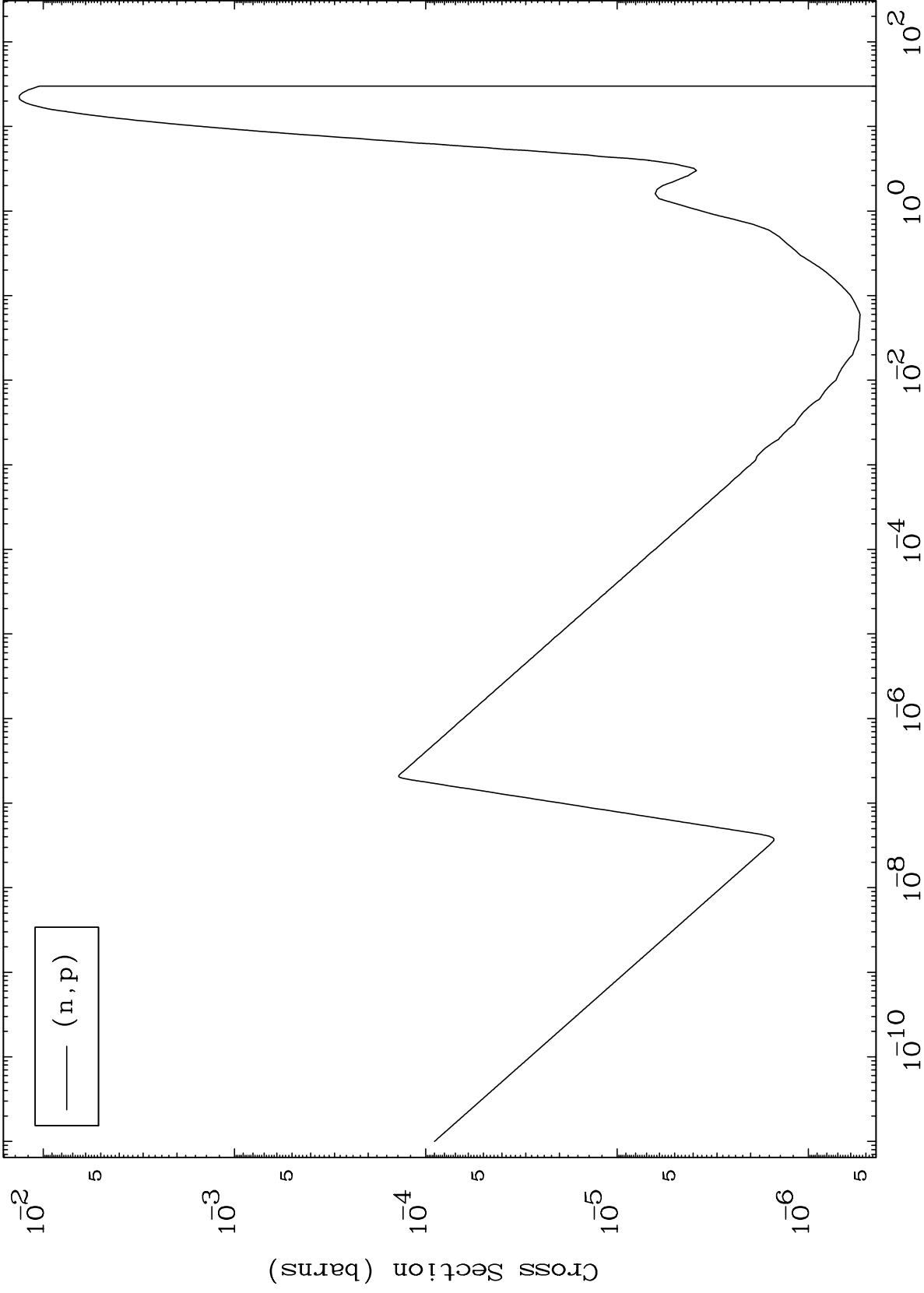
Incident Energy (MeV)

101-Md-249

MAT 9950

(n,p) Levels
293 Kelvin Cross Sections

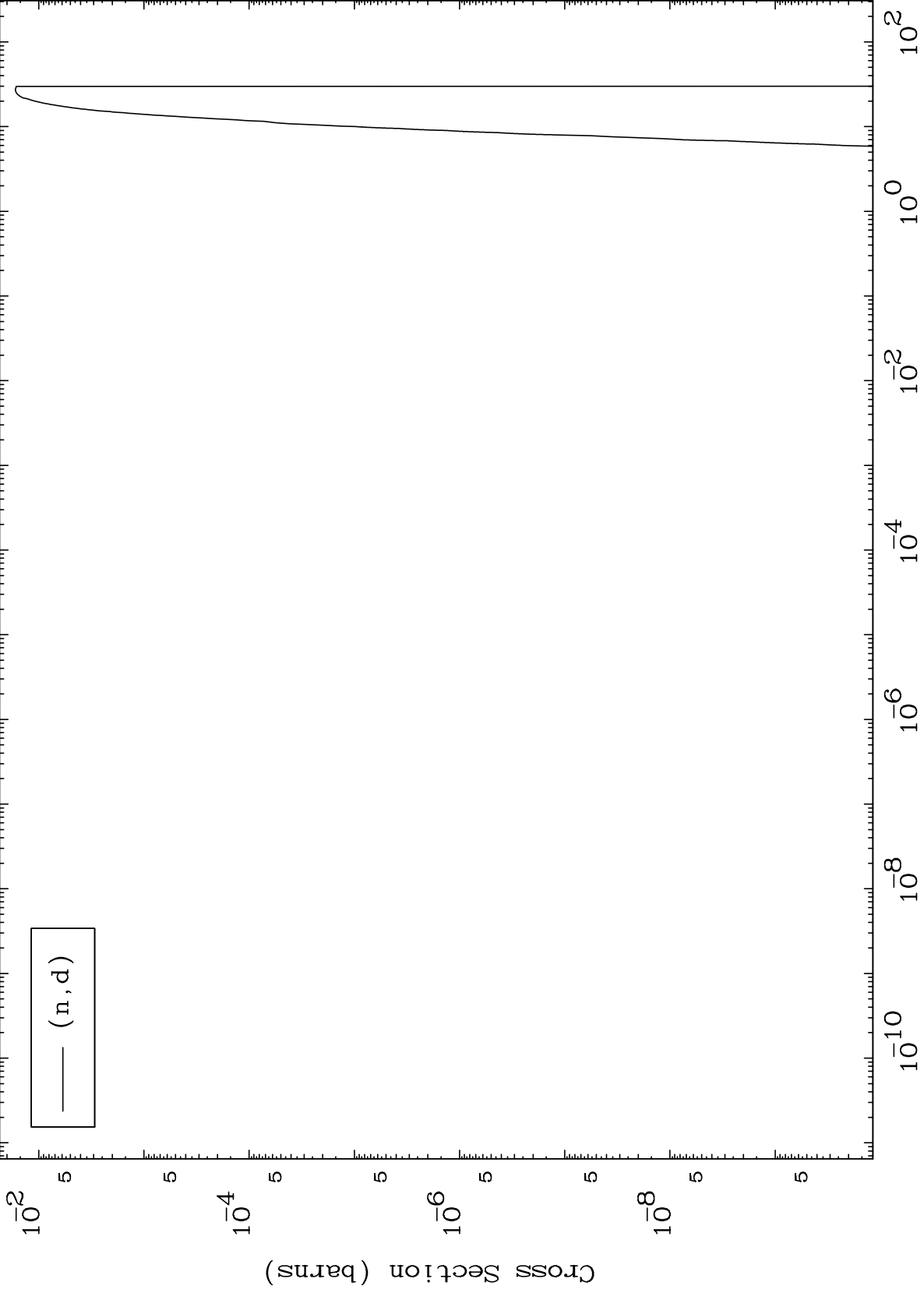
101-Md-249



MAT 9950

(n,d) Levels
293 Kelvin Cross Sections

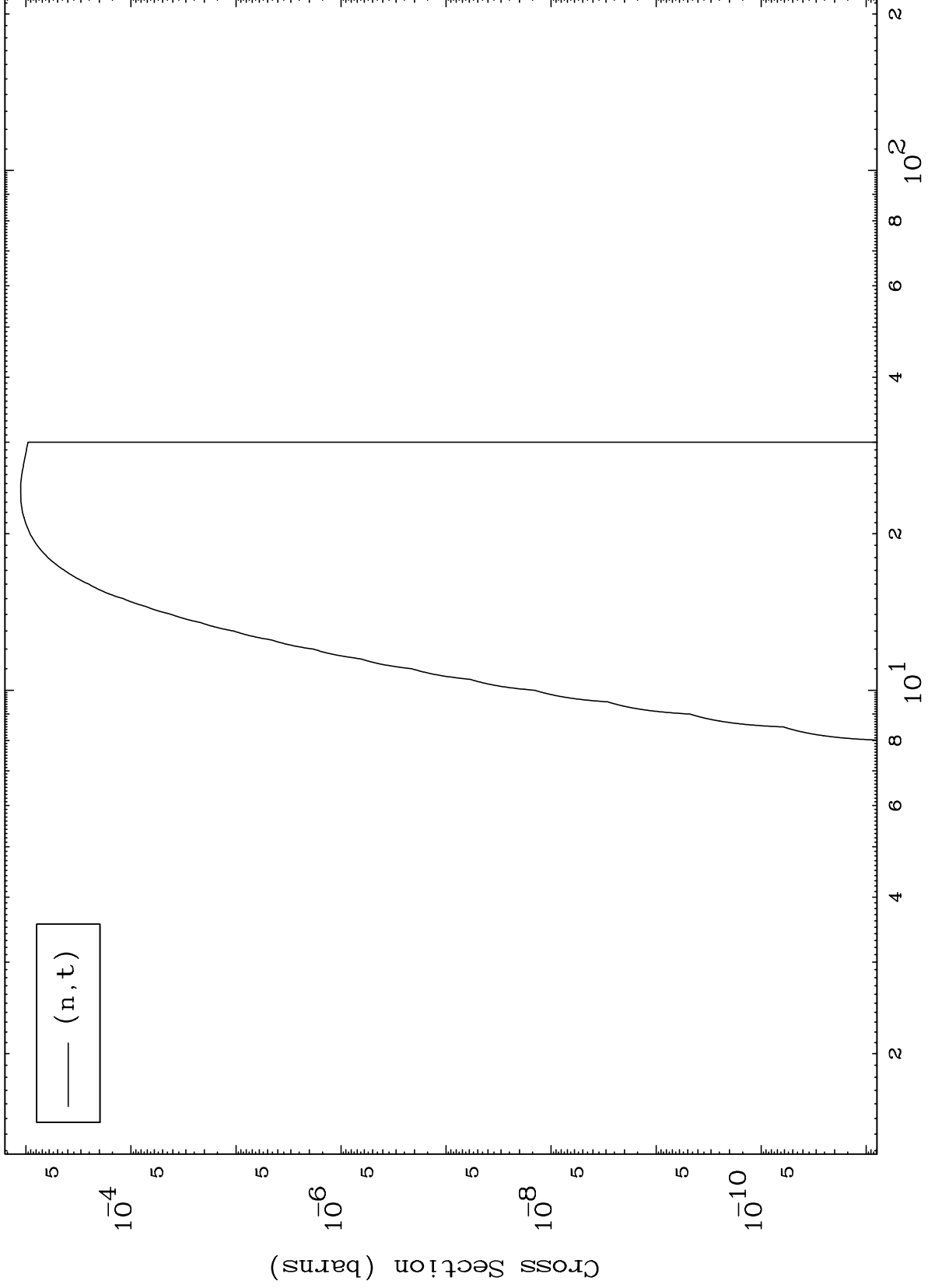
101-Md-249



MAT 9950

(n,t) Levels
293 Kelvin Cross Sections

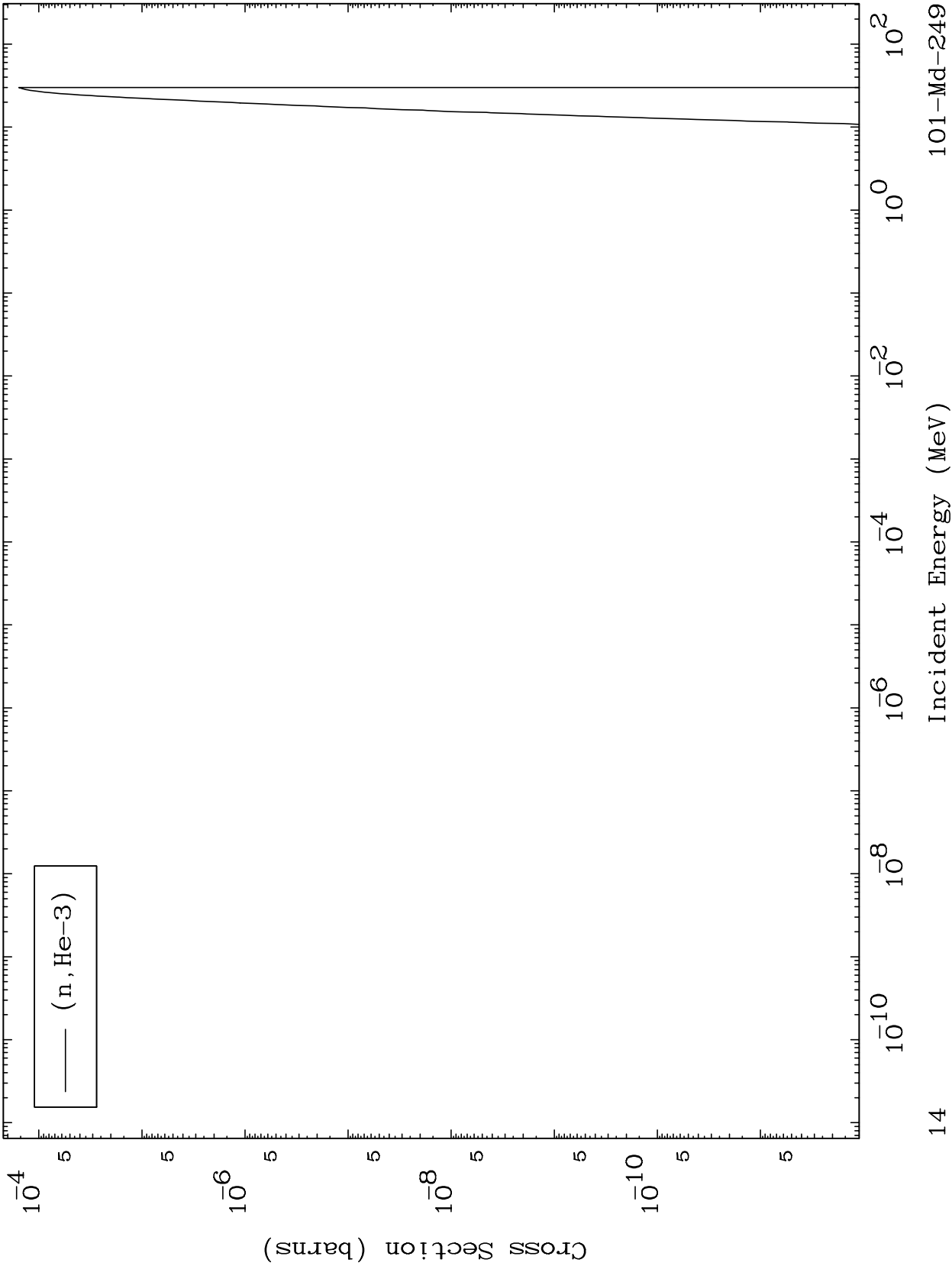
101-Md-249



MAT 9950

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-249

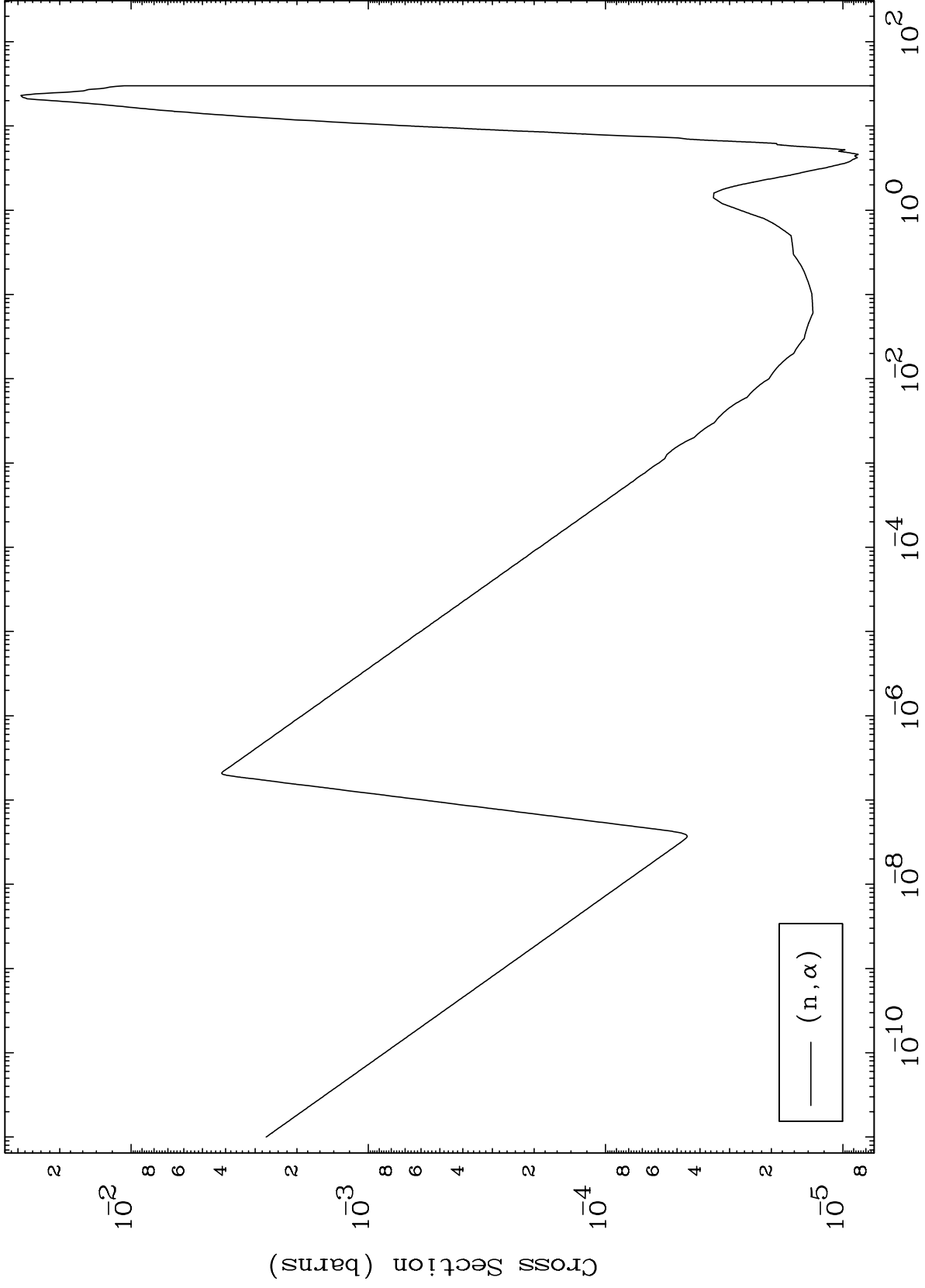


(n, He-3)

MAT 9950

(n, α) Levels
293 Kelvin Cross Sections

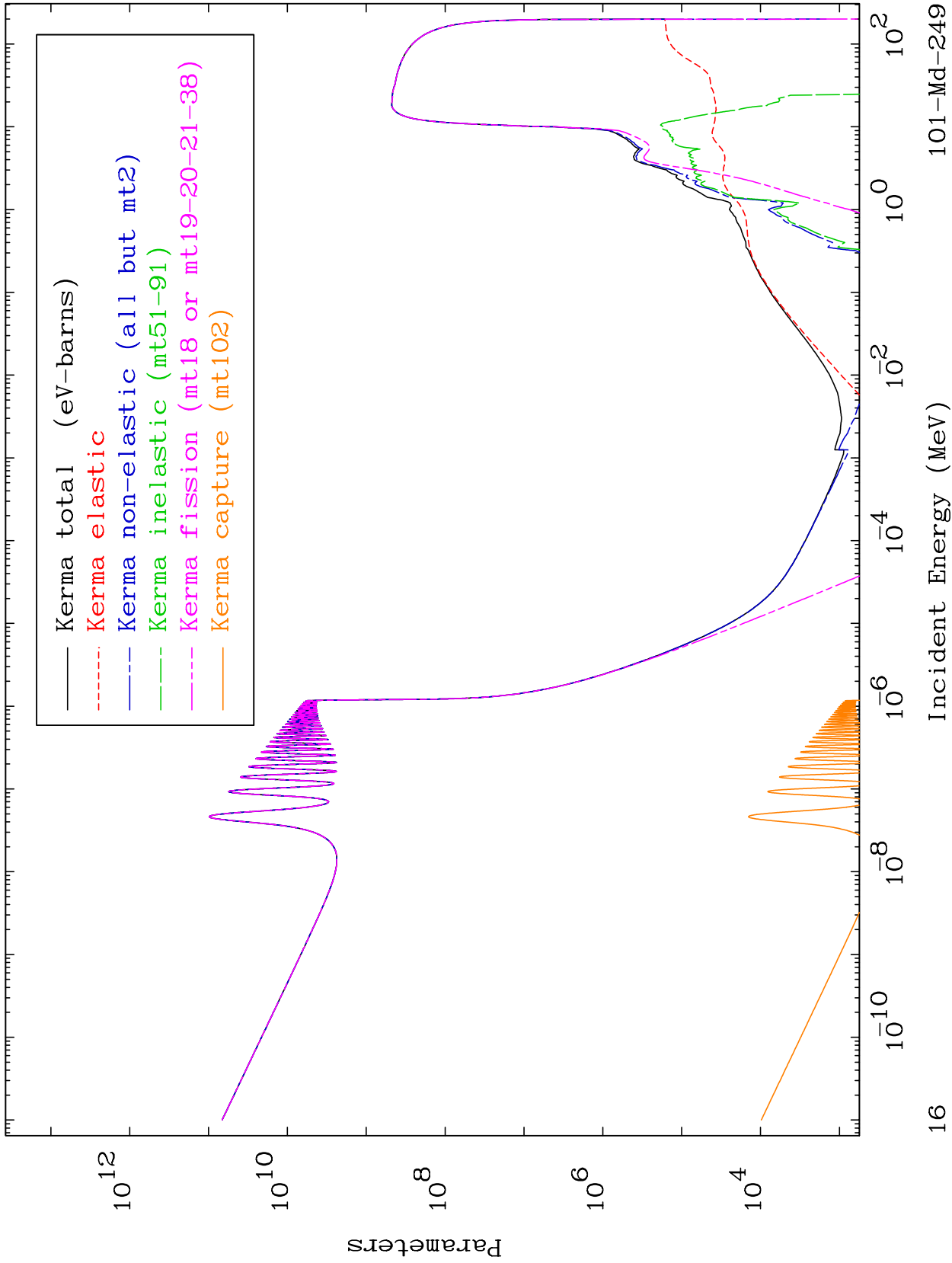
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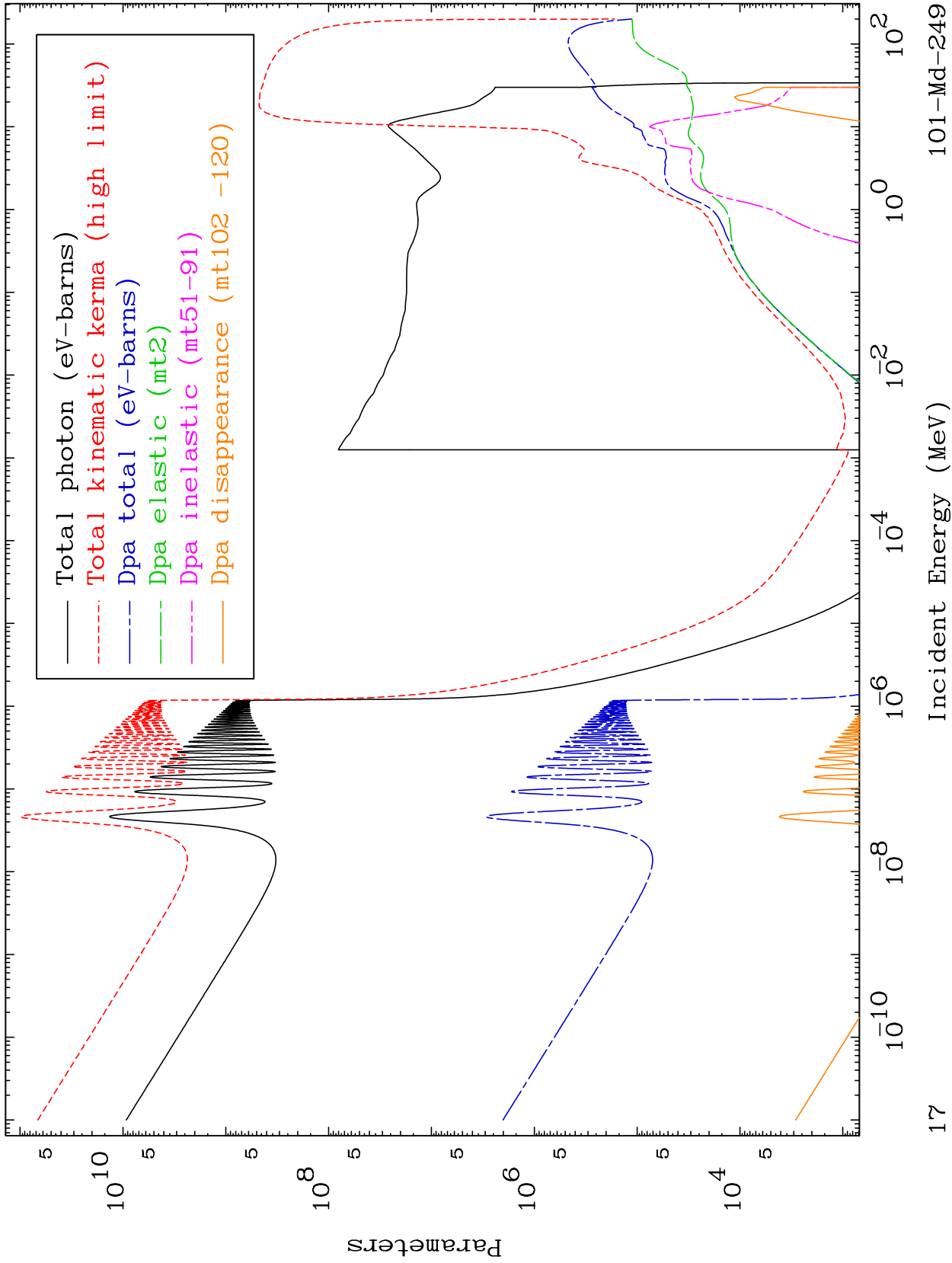


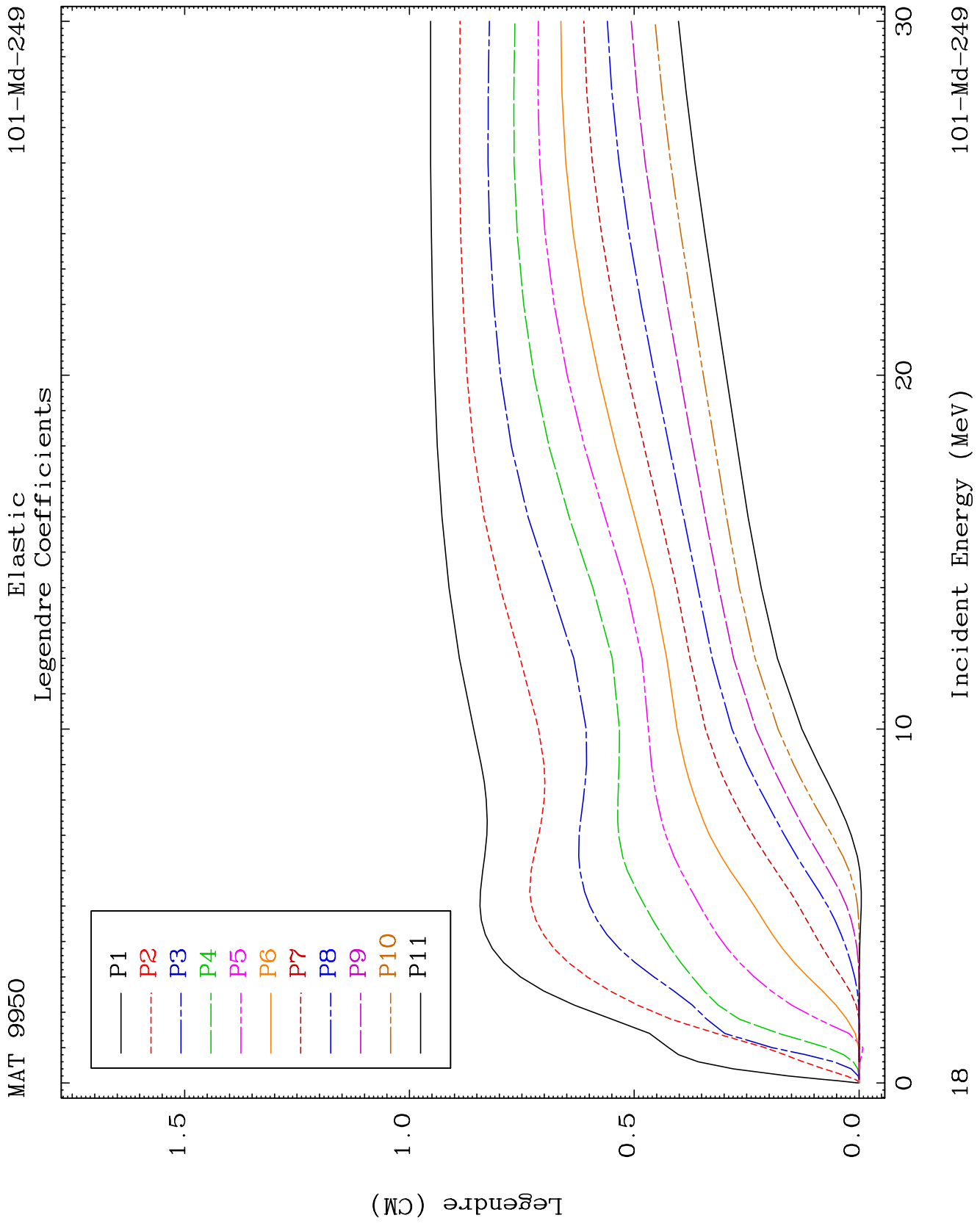
15

Incident Energy (MeV)

101-Md-249



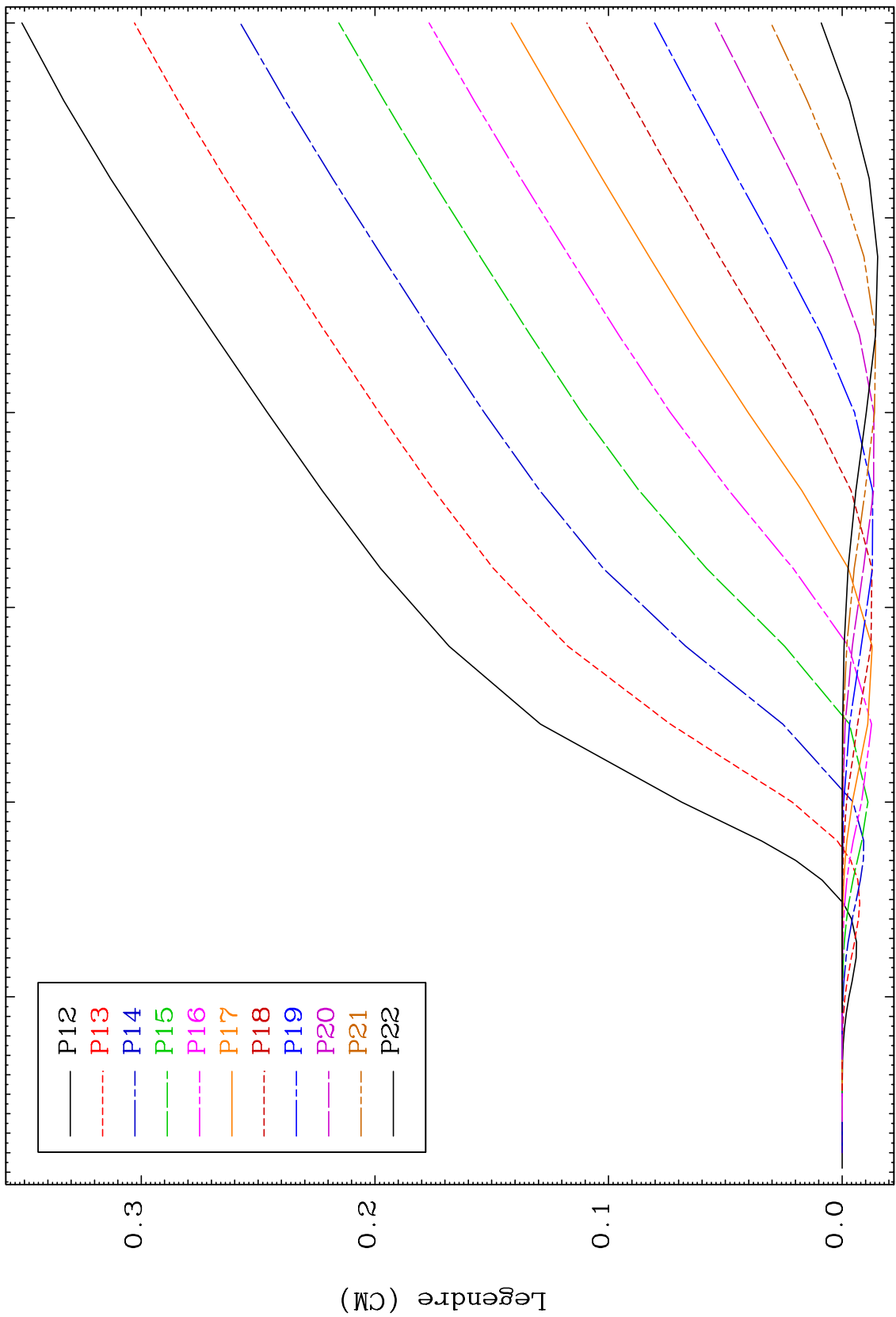




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Elastic Legendre Coefficients

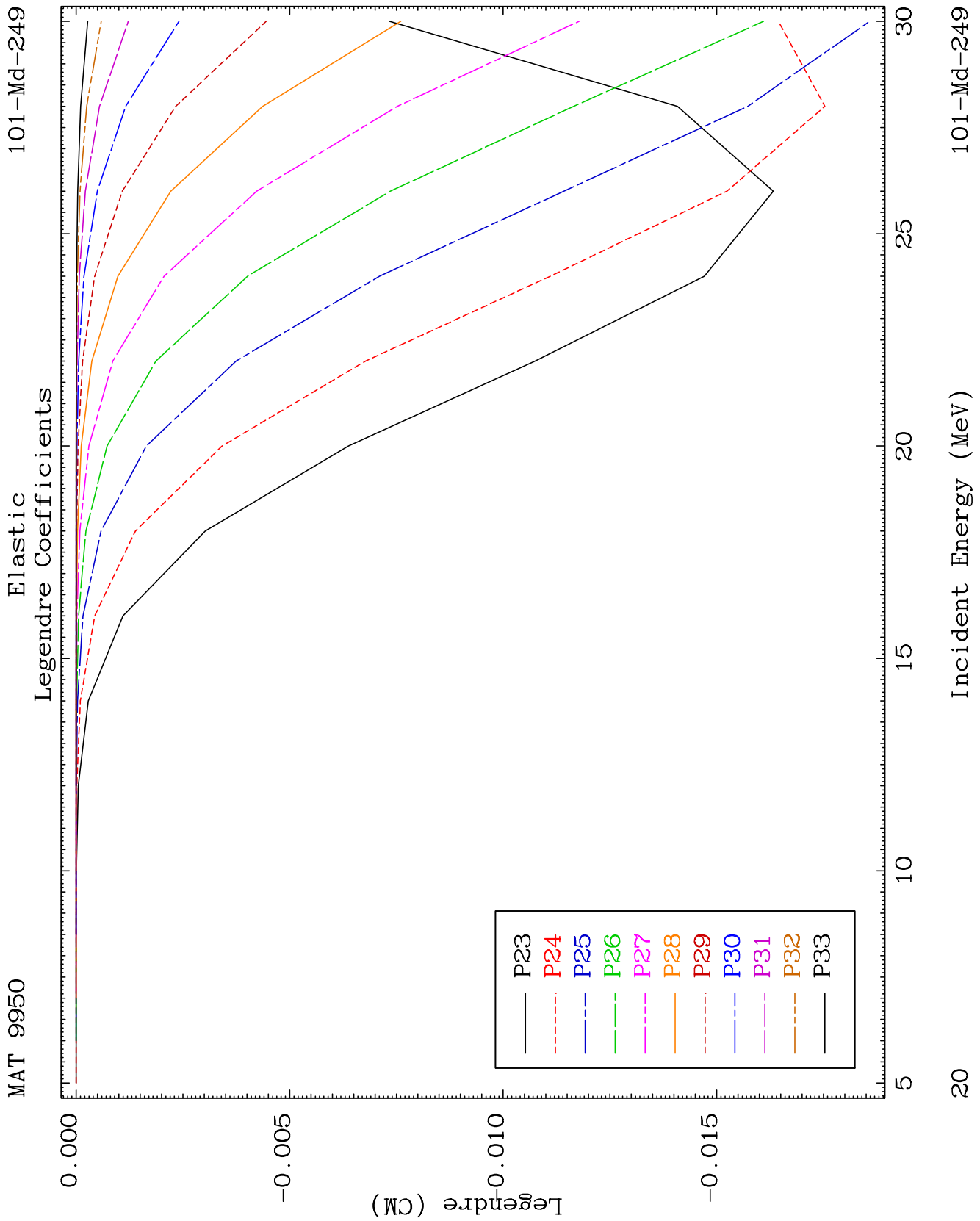
101-Md-249



19

Incident Energy (MeV)

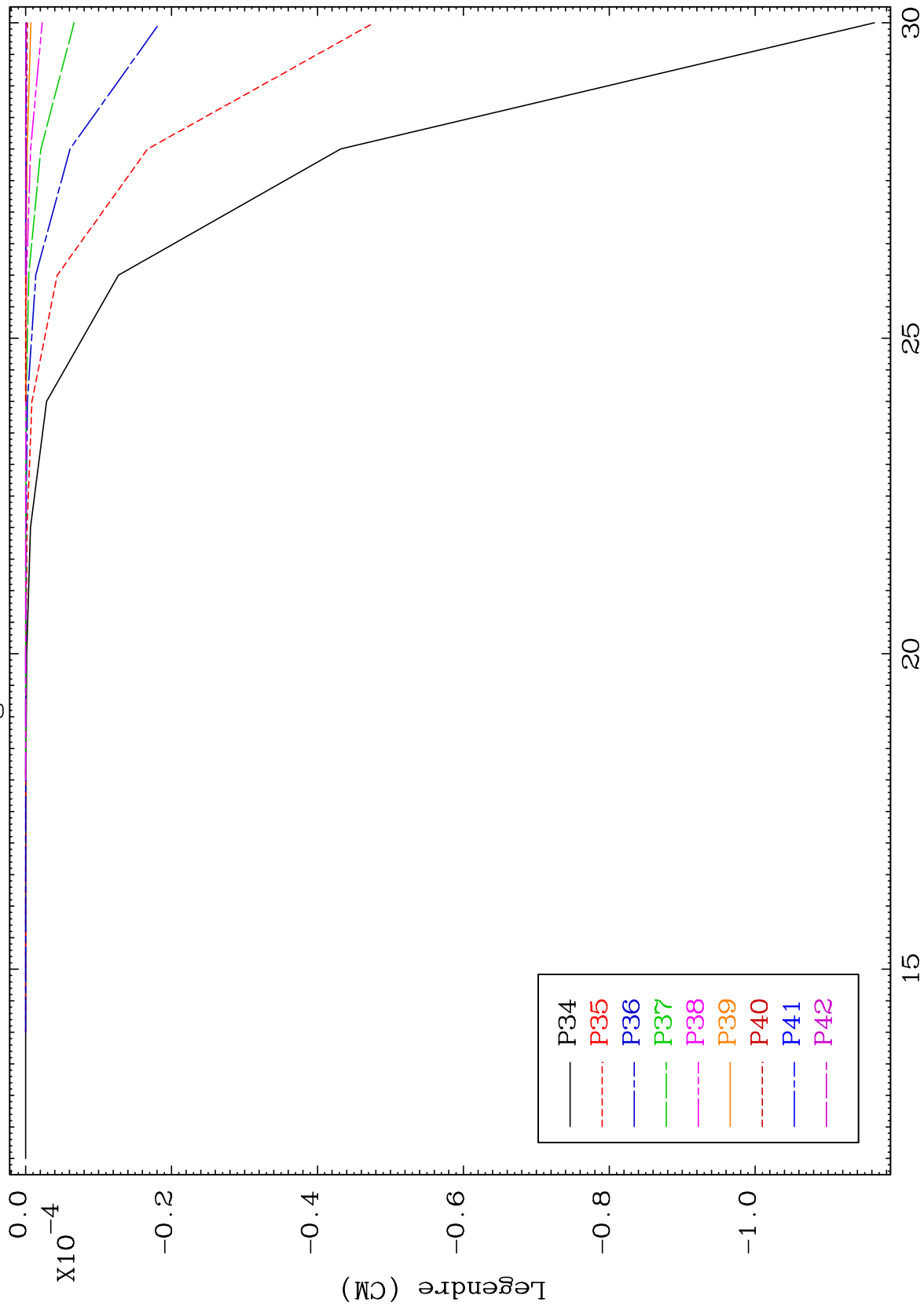
101-Md-249



MAT 9950

Elastic Legendre Coefficients

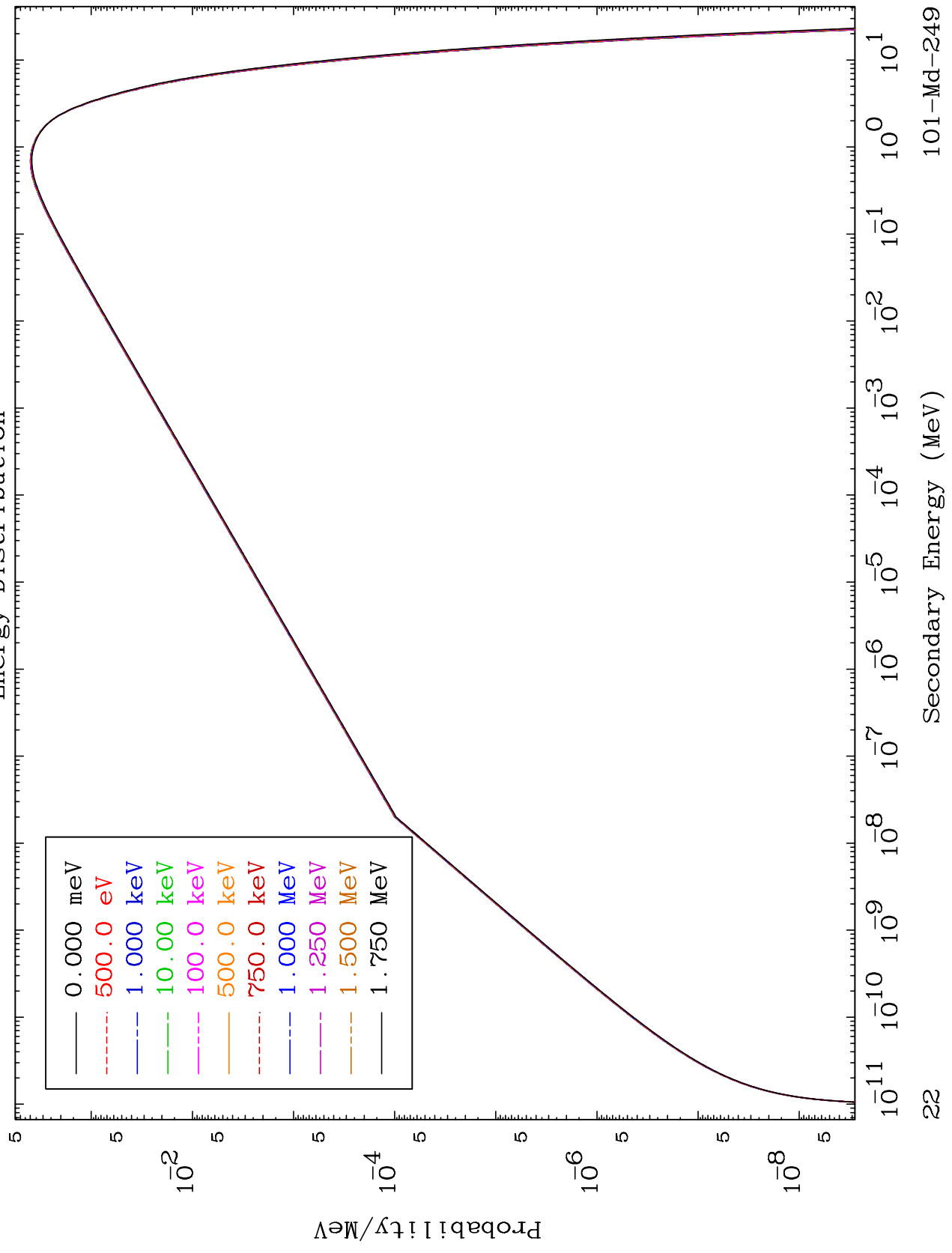
101-Md-249



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Fission Energy Distribution

101-Md-249



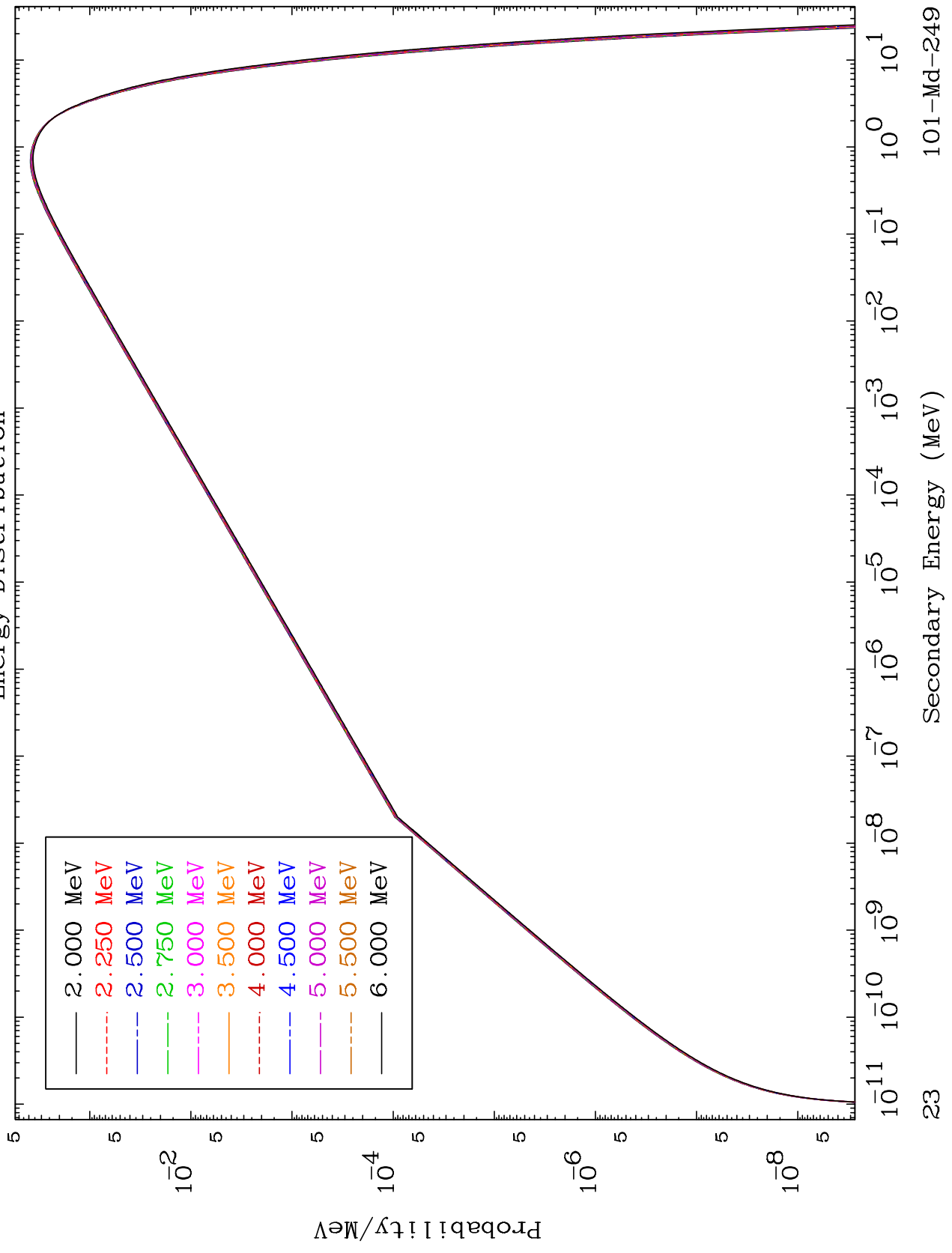
101-Md-249

Secondary Energy (MeV)

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Fission Energy Distribution

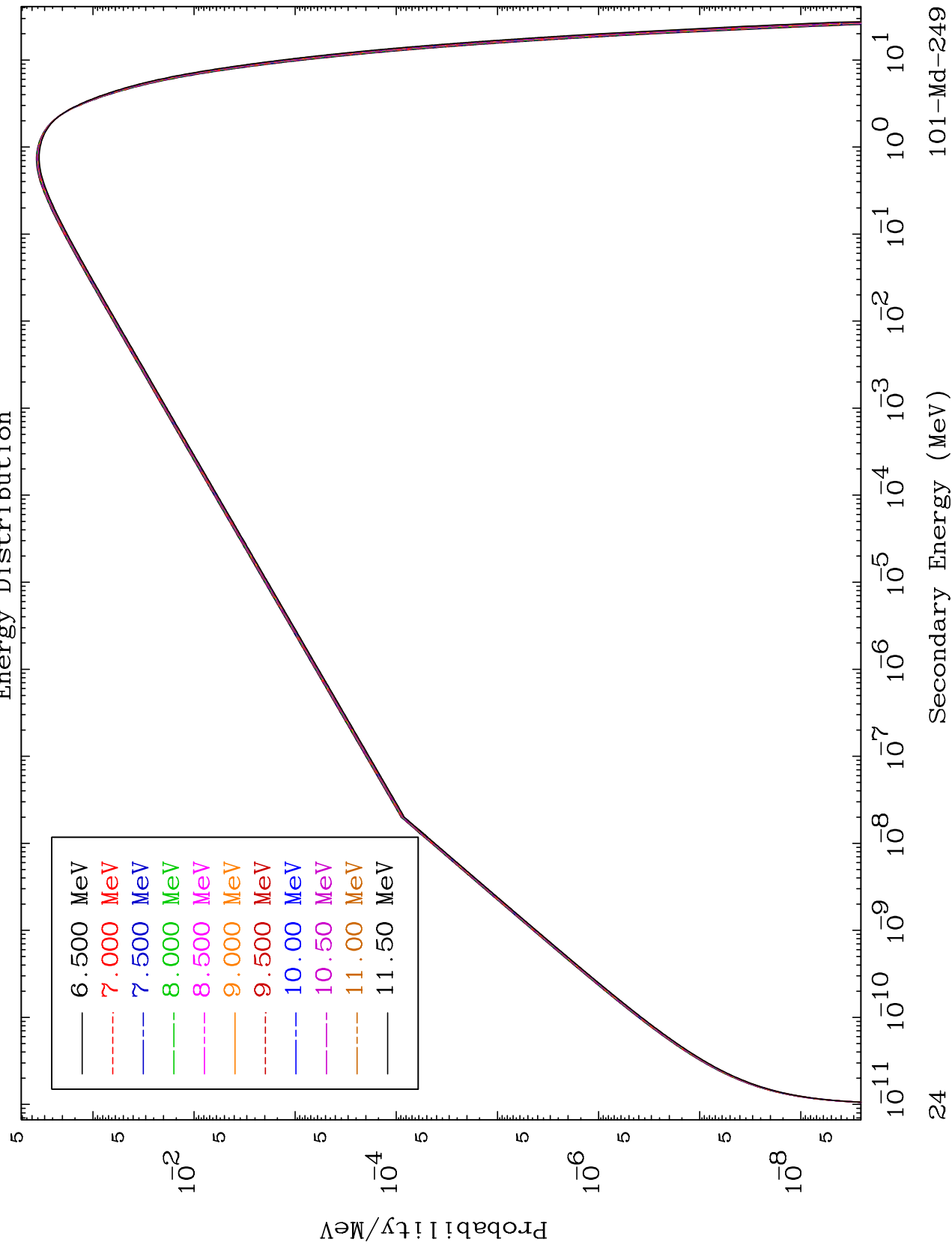
101-Md-249



MAT 9950

Fission Energy Distribution

101-Md-249



101-Md-249

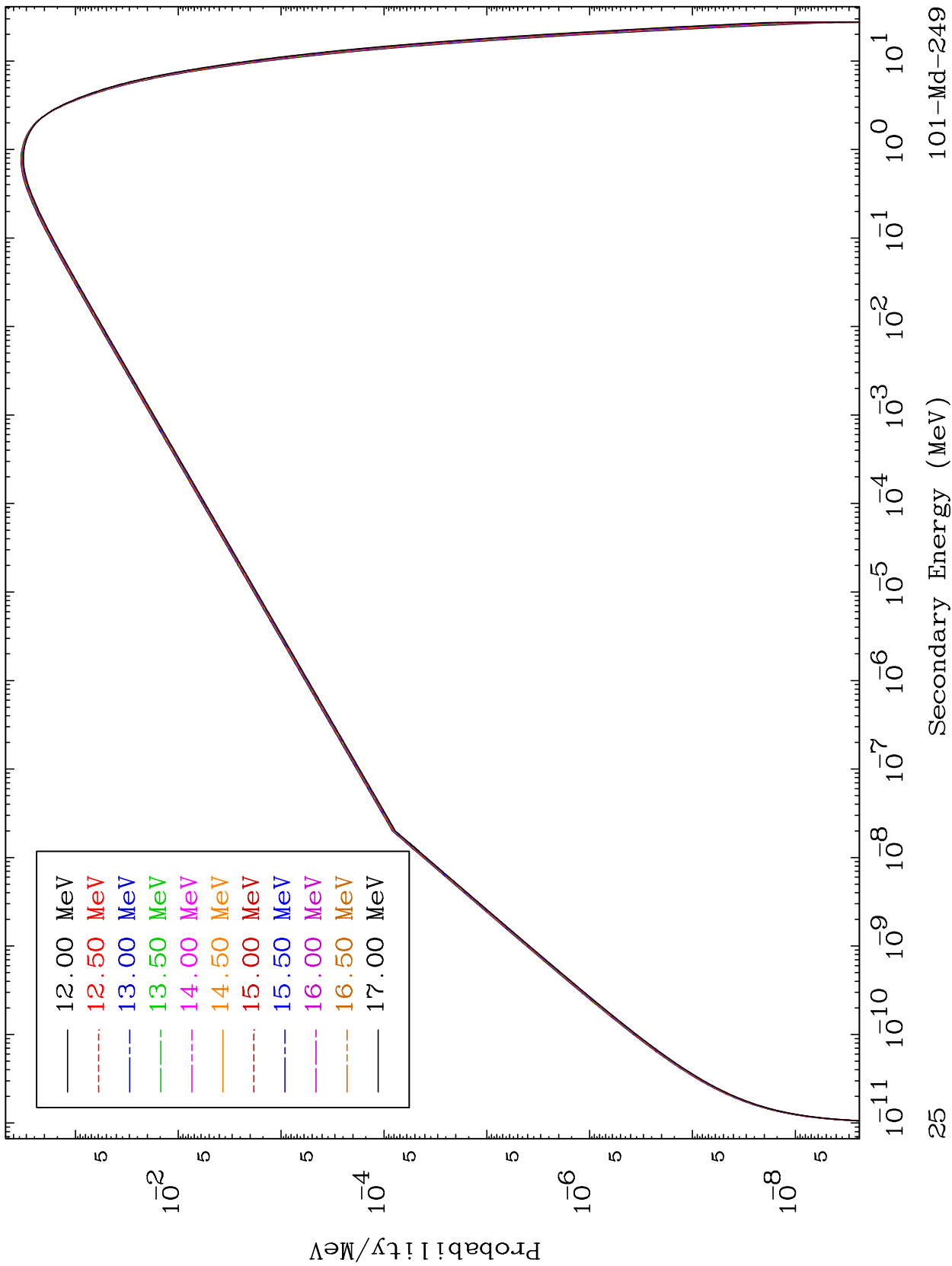
Secondary Energy (MeV)

24

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Fission Energy Distribution

101-Md-249



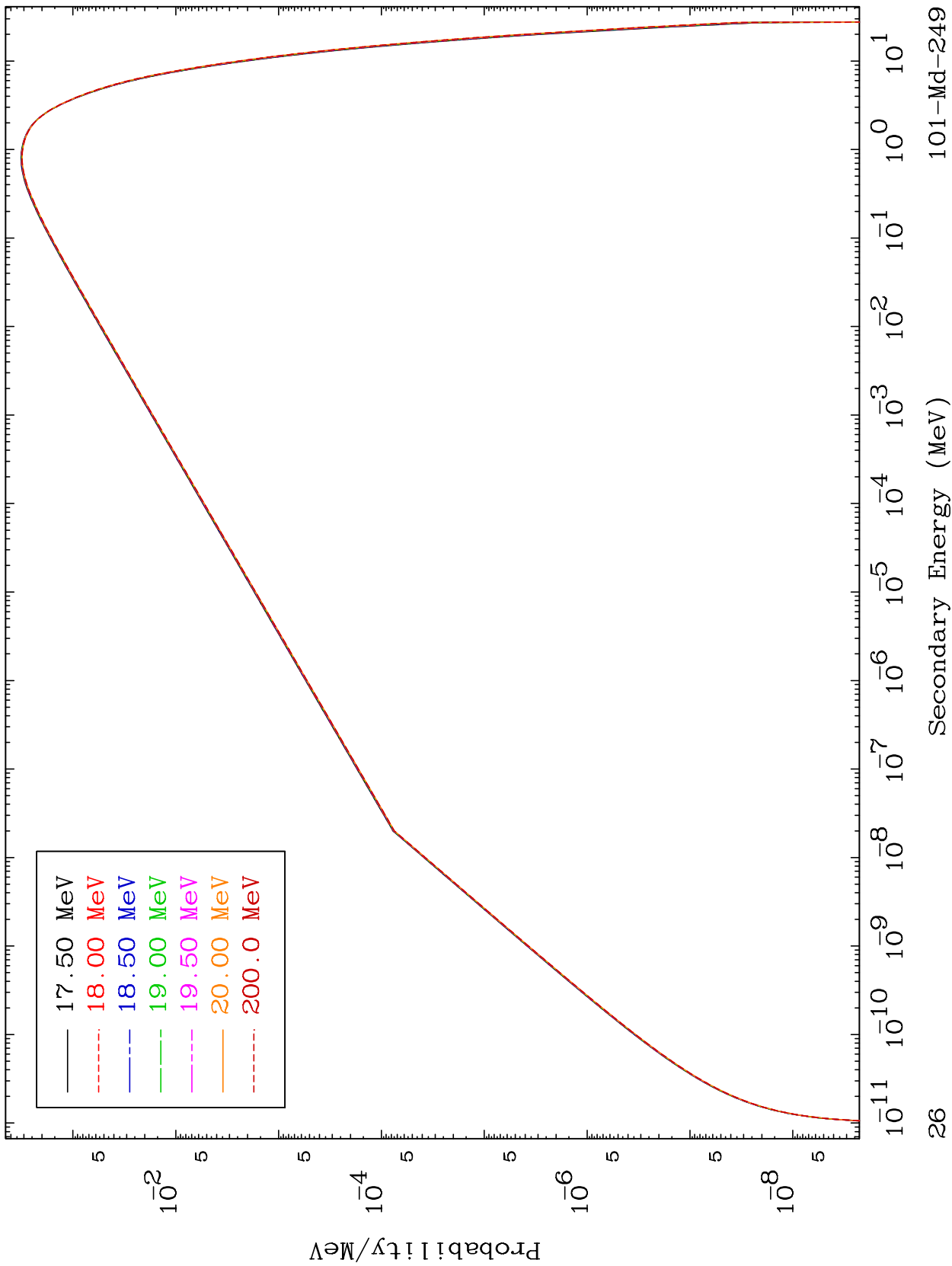
101-Md-249

Secondary Energy (MeV)

MAT 9950

Fission
Energy Distribution

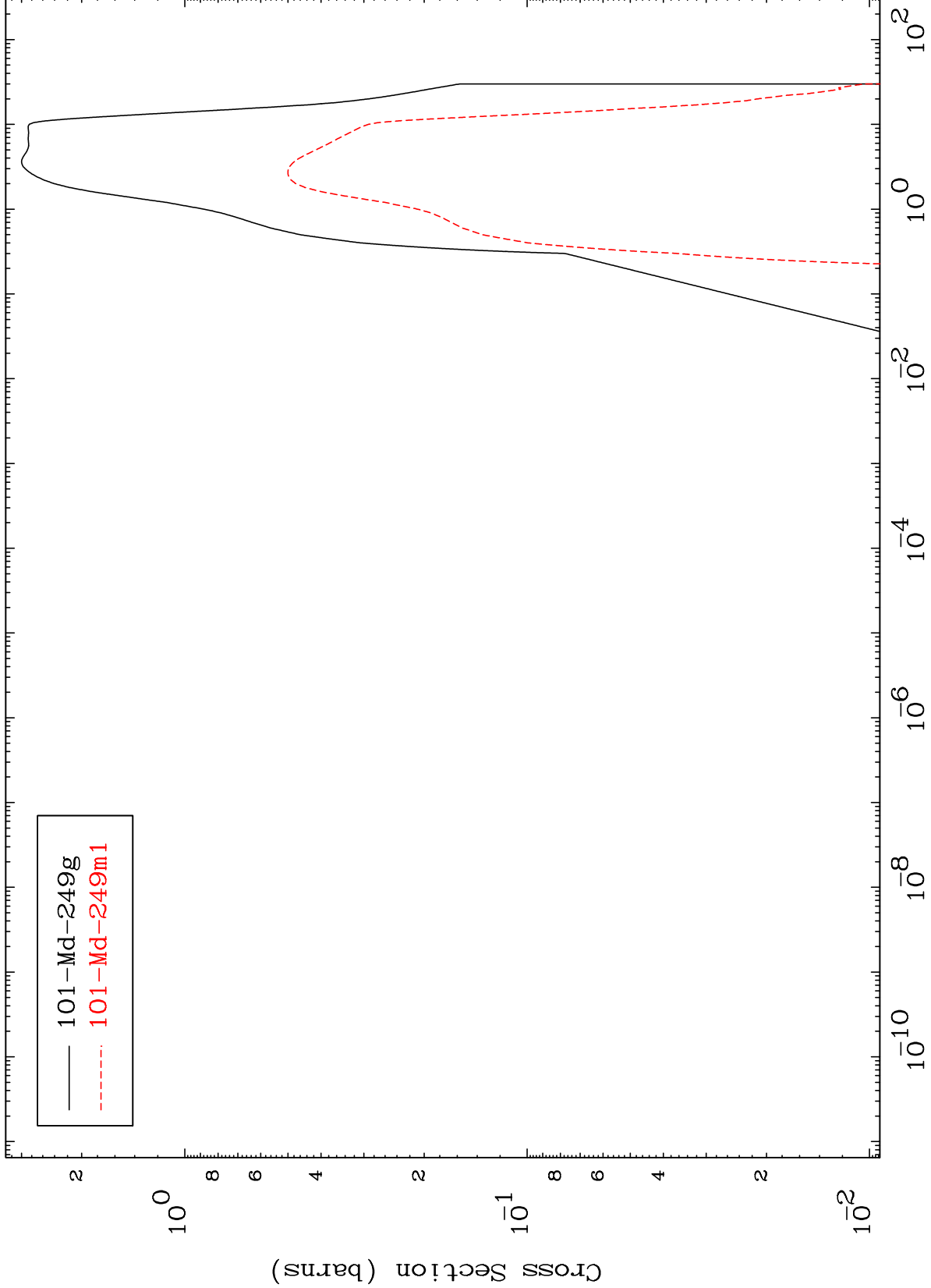
101-Md-249



MAT 9950

Inelastic
Radionuclide Production Cross Section

101-Md-249



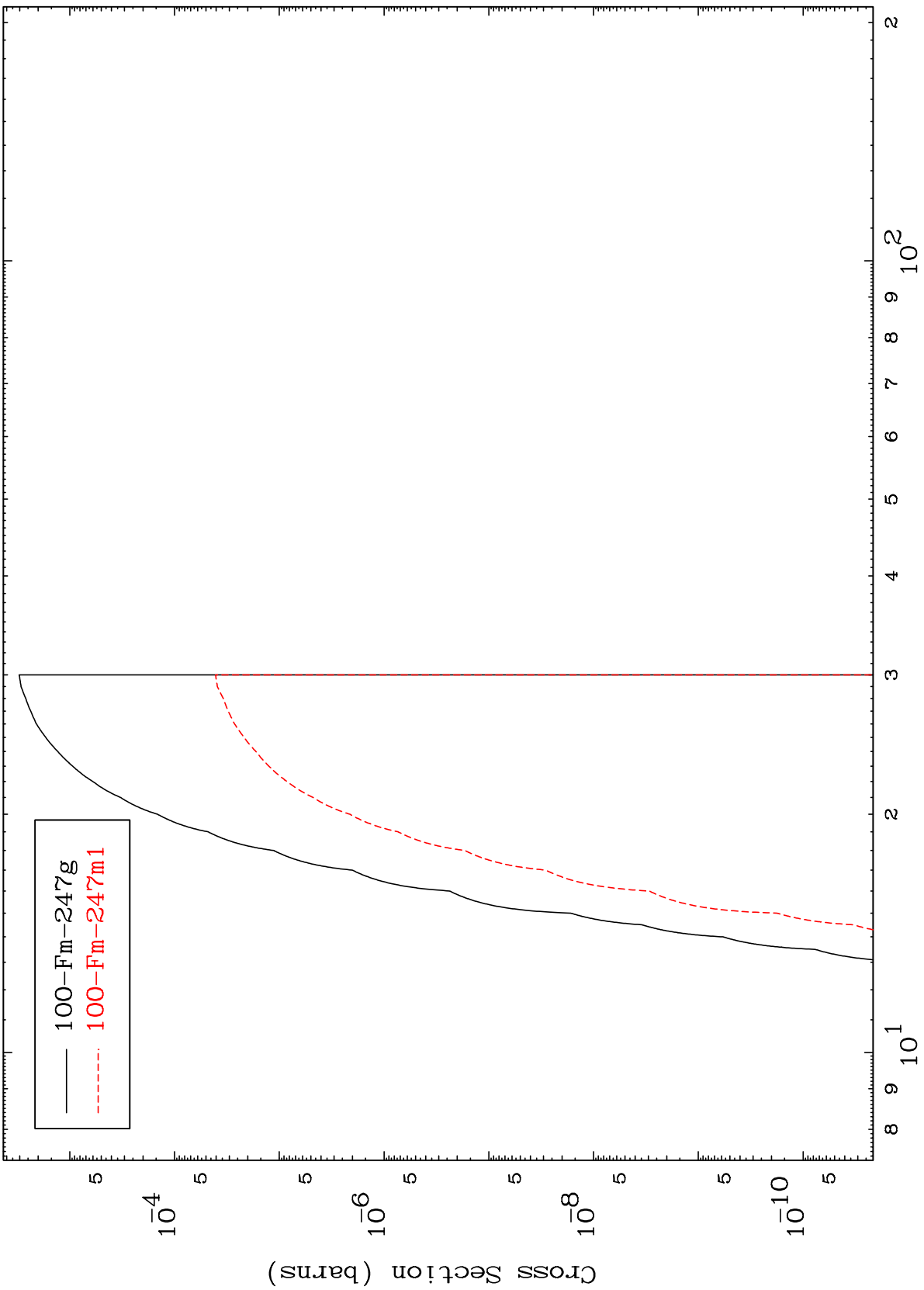
— 101-Md-249g
- - - 101-Md-249m1

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(n,n') d

101-Md-249

Radionuclide Production Cross Section



28

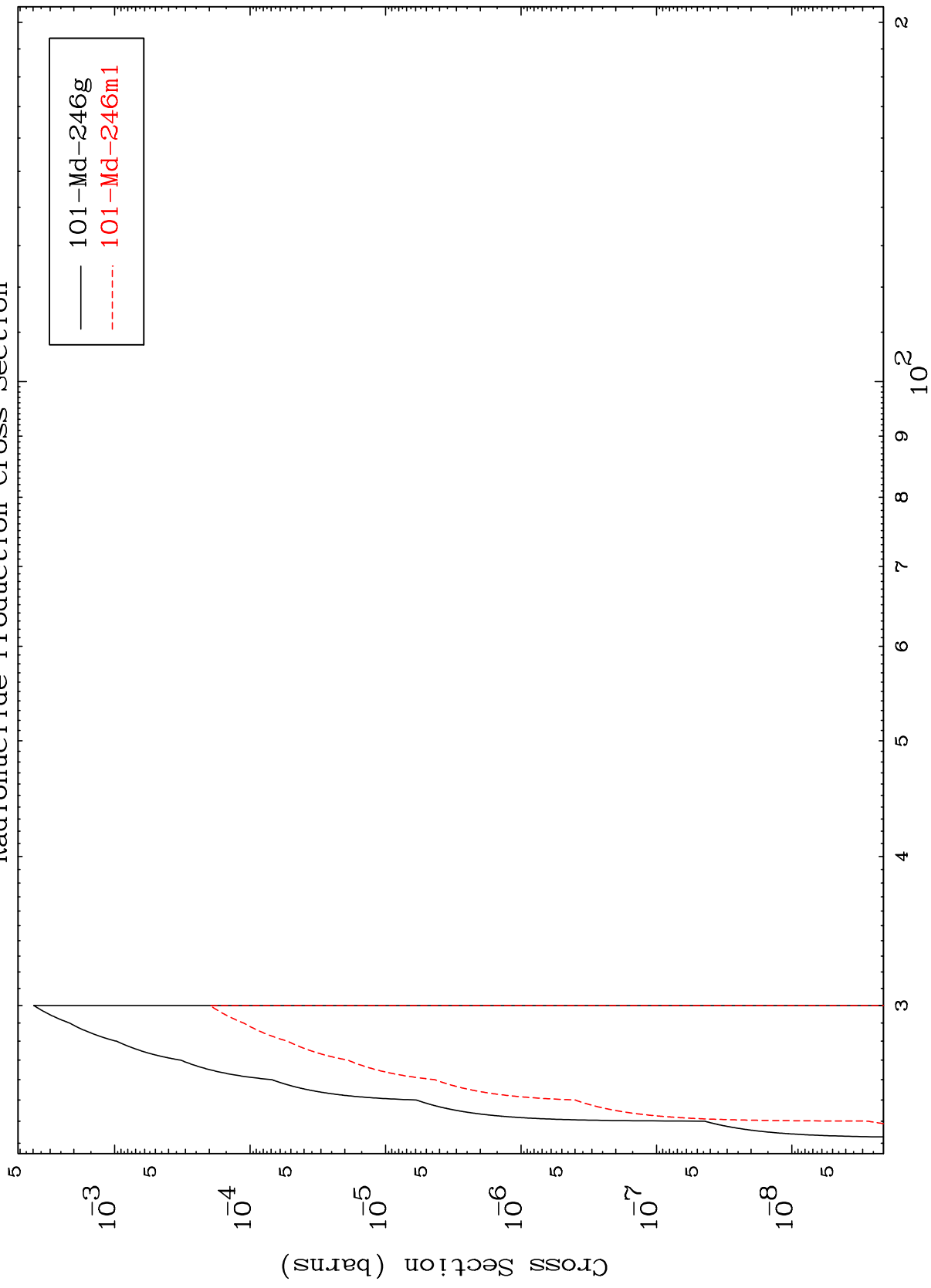
Incident Energy (MeV)

101-Md-249

MAT 9950

101-Md-249

(n,4n)
Radionuclide Production Cross Section



29

101-Md-249

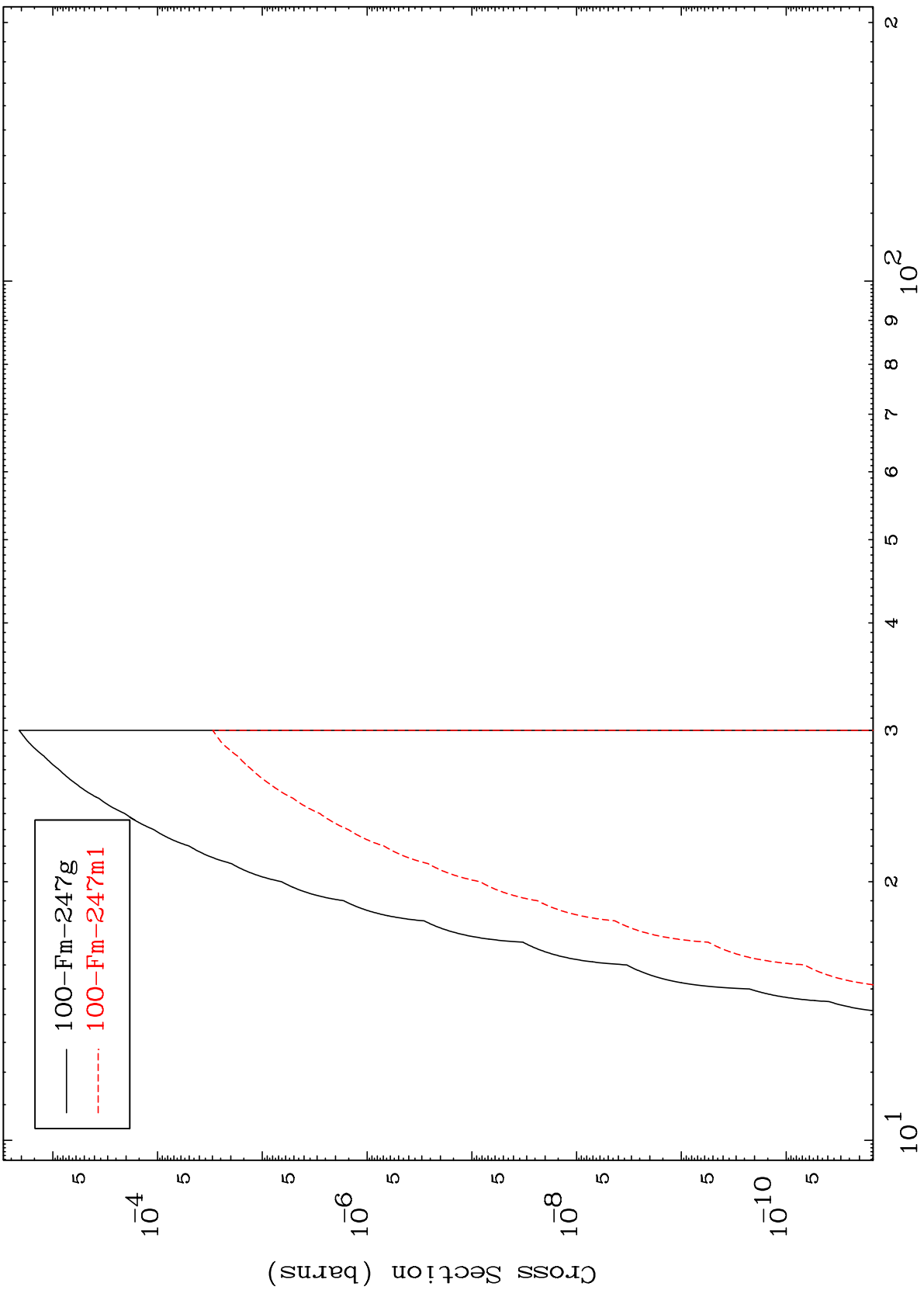
Incident Energy (MeV)

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(n,2n) p

101-Md-249

Radionuclide Production Cross Section



Incident Energy (MeV)

101-Md-249

MAT 9950

101-Md-249

(n, t)
Radionuclide Production Cross Section

