

Program EVALPLOT
(Version 2018-1)

by

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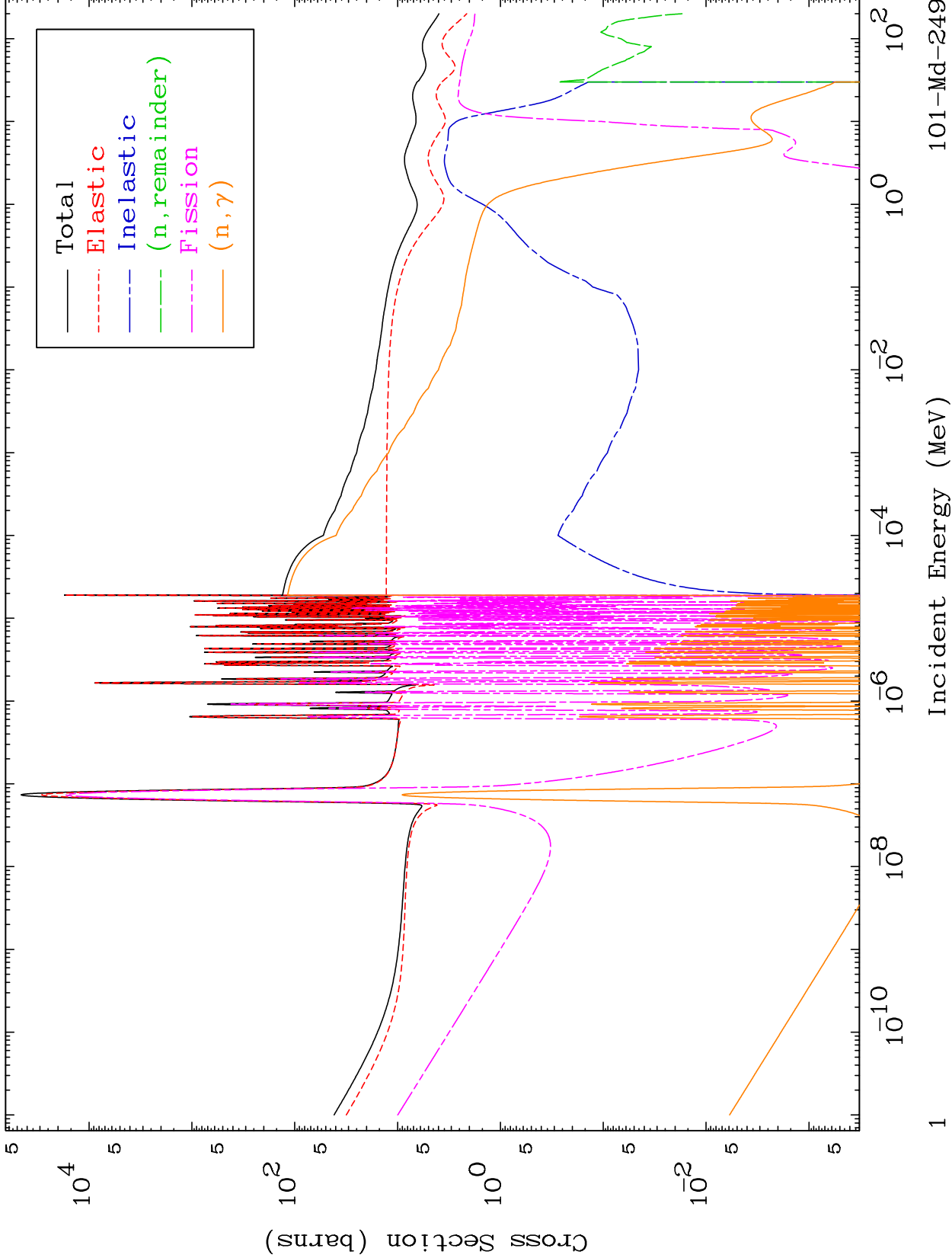
Press Mouse Button to Start

MAT 9955

Major

293 Kelvin Cross Sections

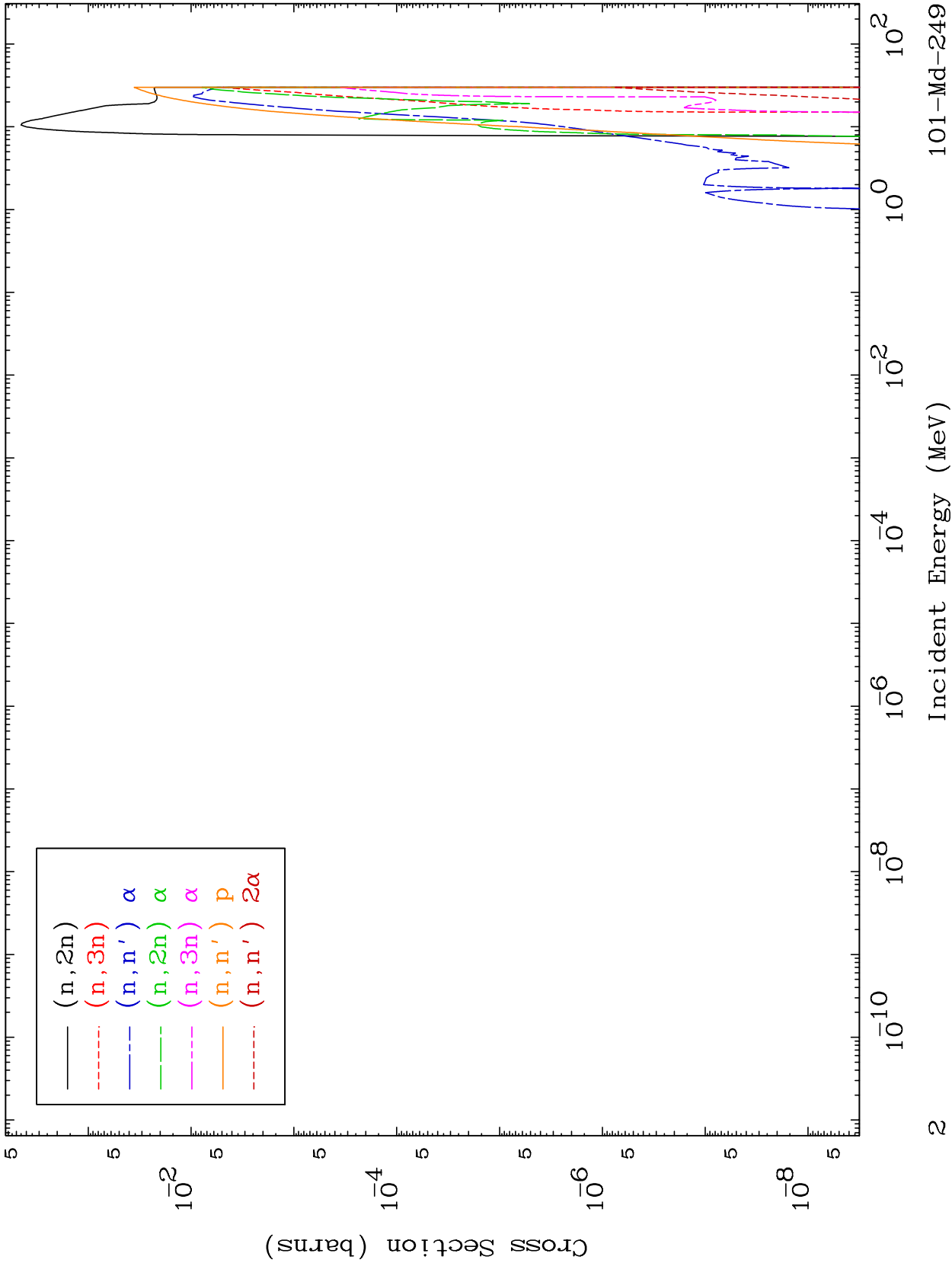
101-Md-249



MAT 9955

Neutron Production
293 Kelvin Cross Sections

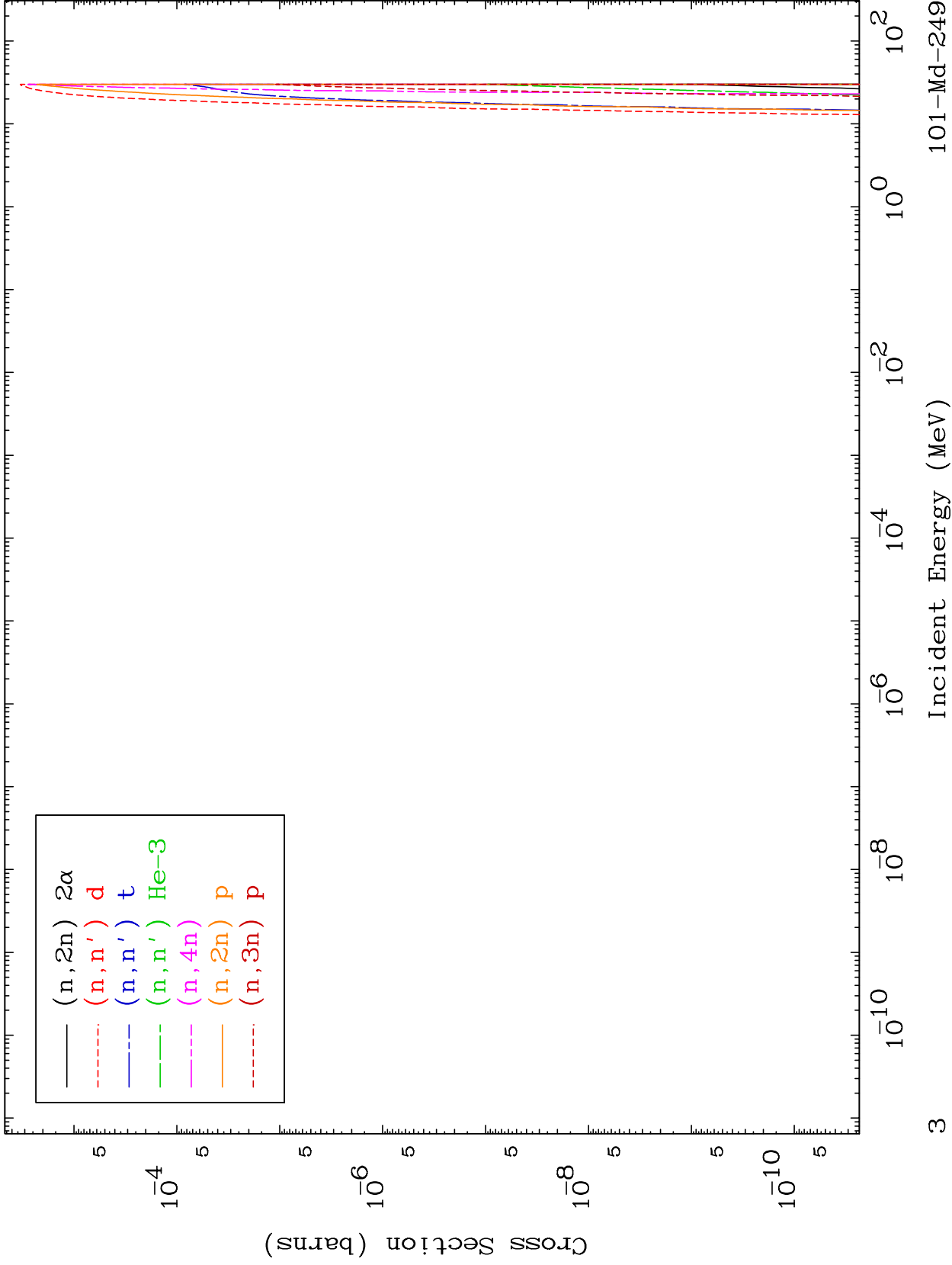
101-Md-249



MAT 9955

Neutron Production
293 Kelvin Cross Sections

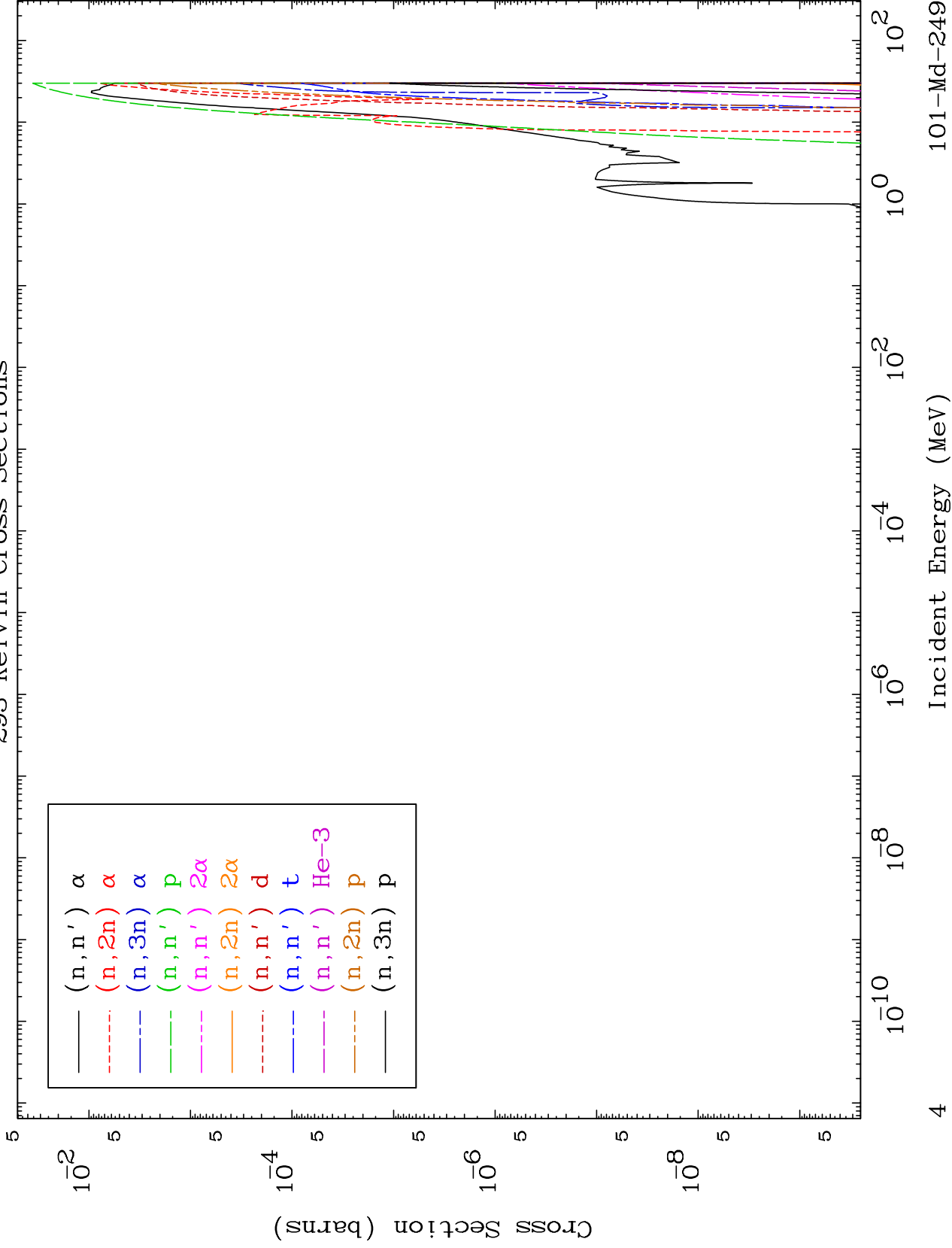
101-Md-249



MAT 9955

Charged Particle
293 Kelvin Cross Sections

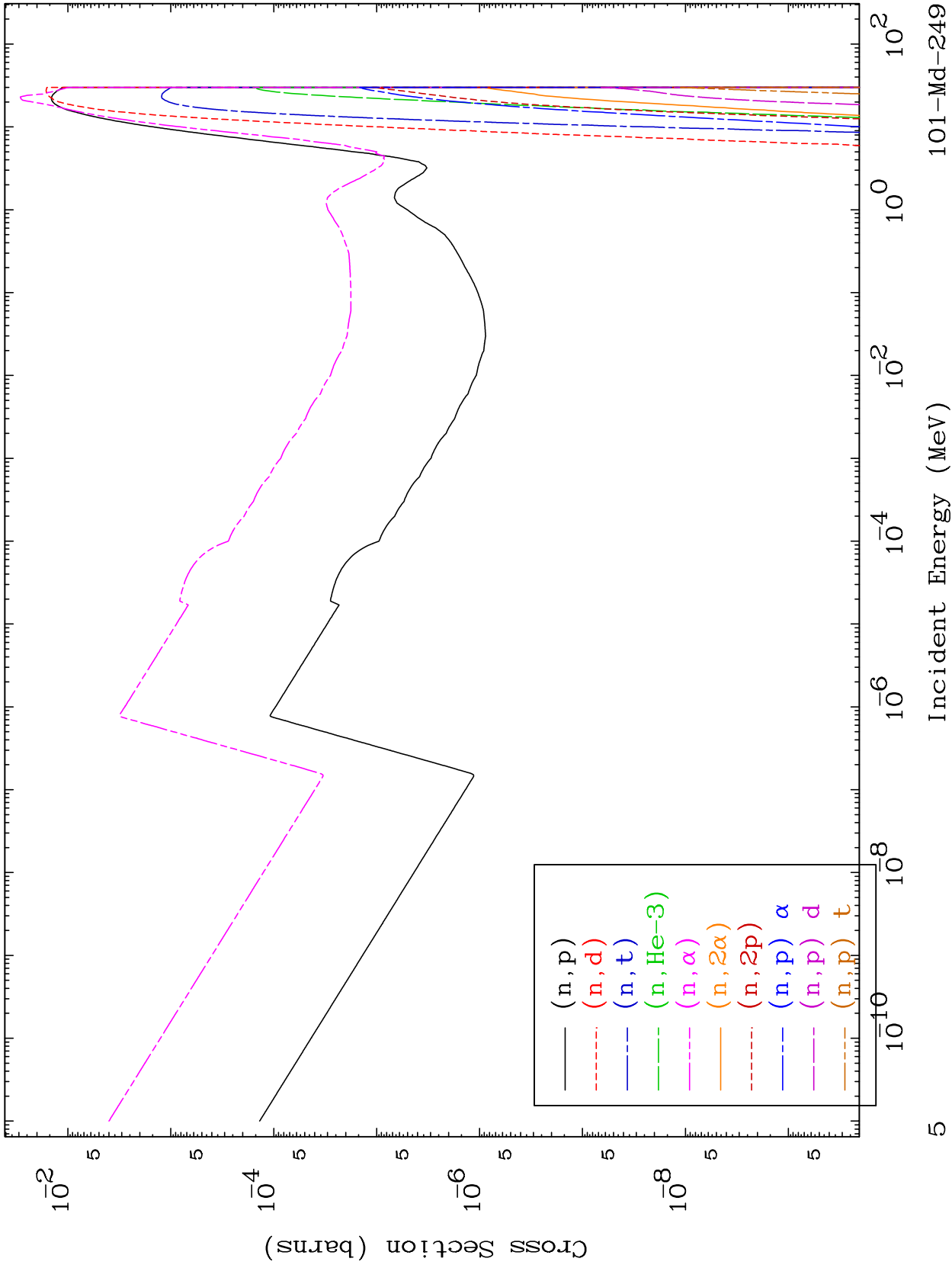
101-Md-249



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Charged Particle
293 Kelvin Cross Sections

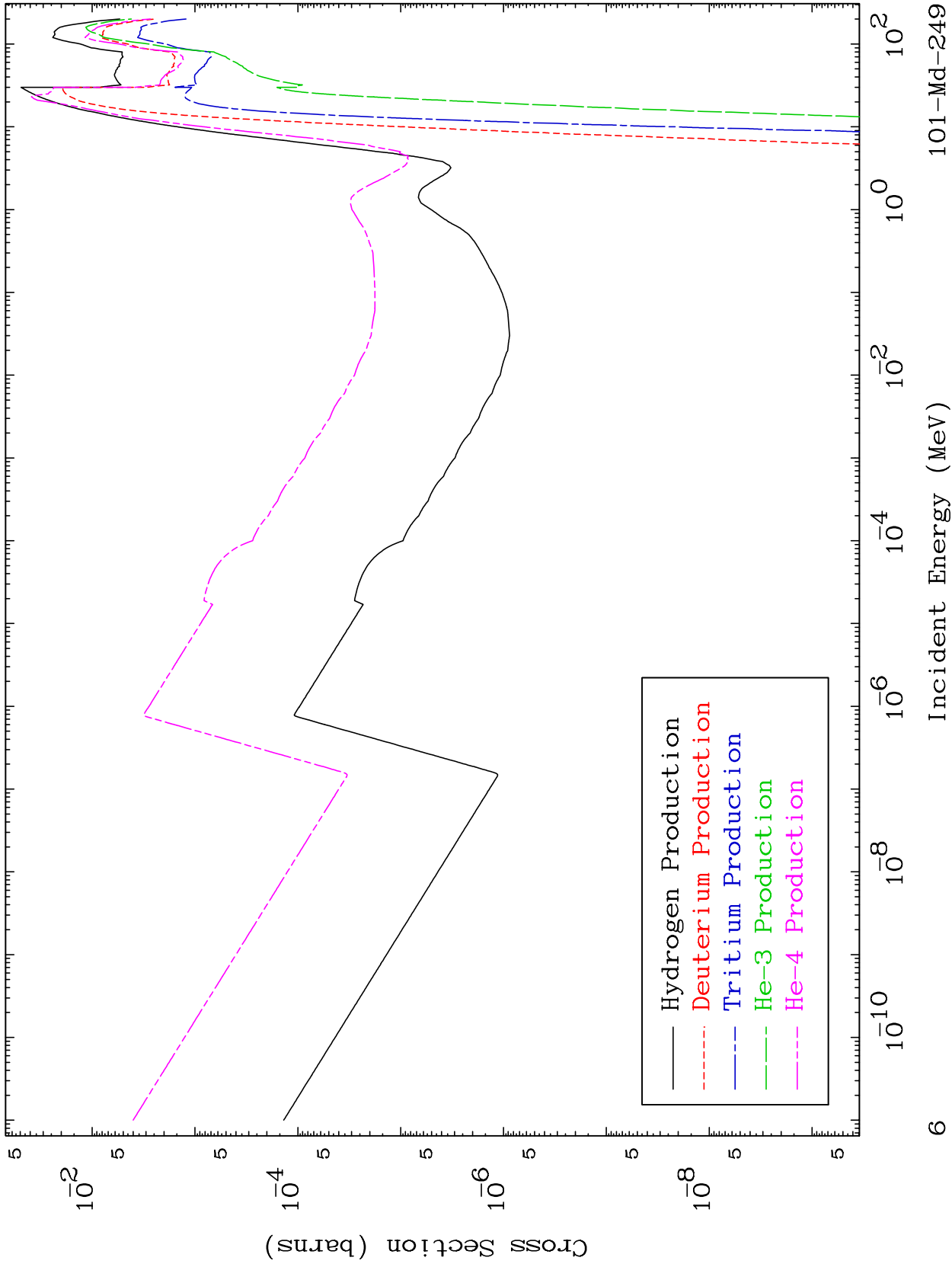
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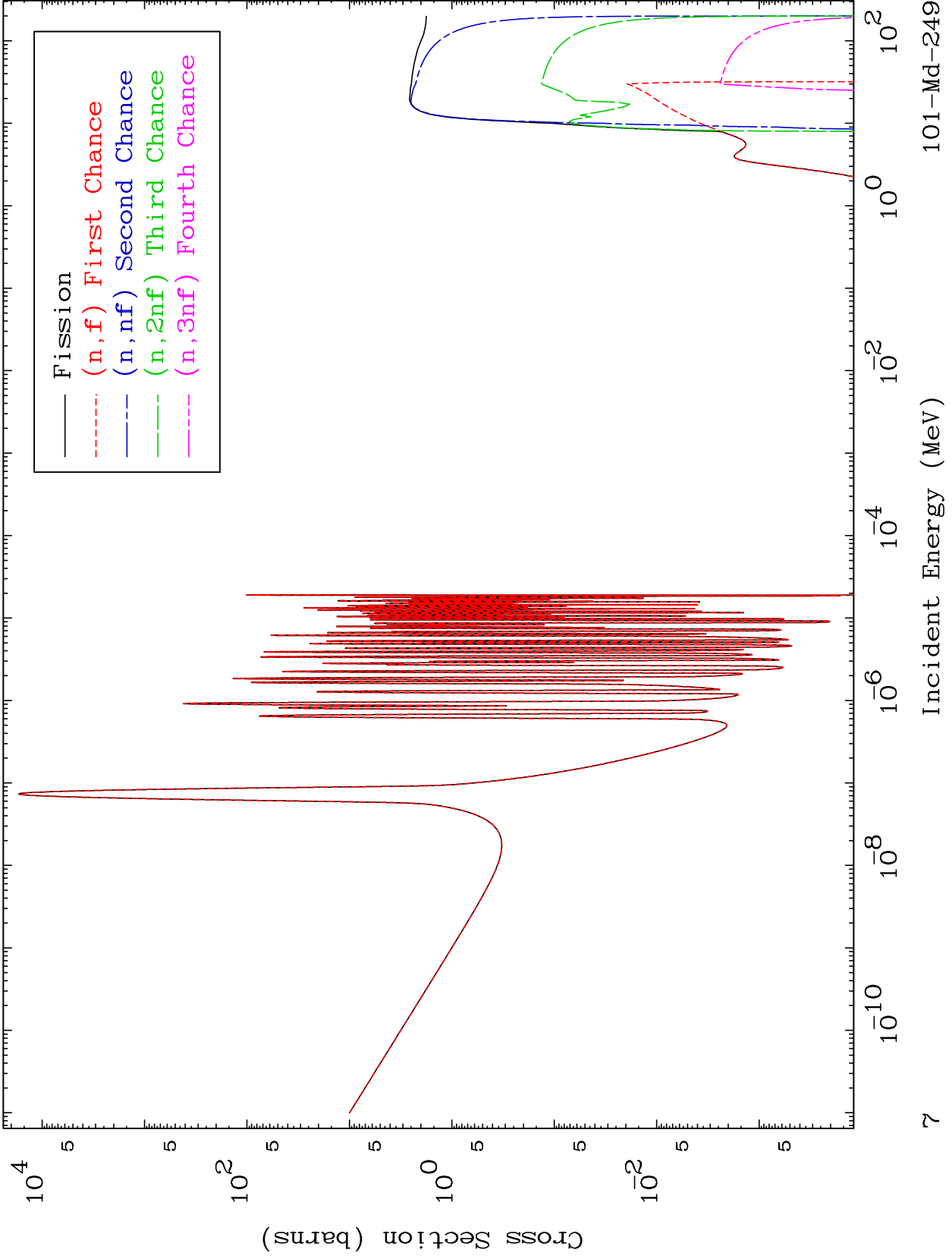


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Particle Production
293 Kelvin Cross Sections

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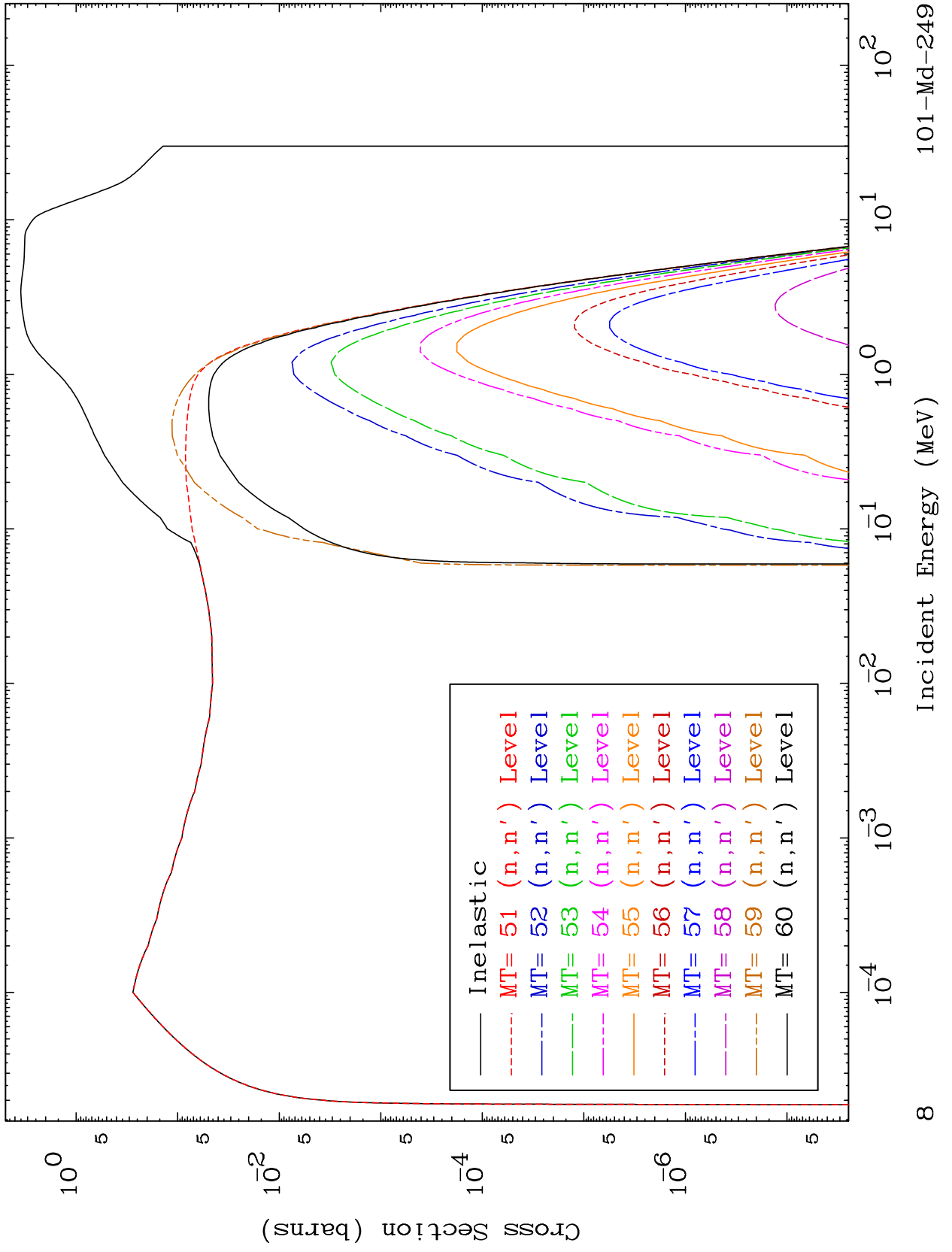




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(n,n') Level
293 Kelvin Cross Sections

101-Md-249

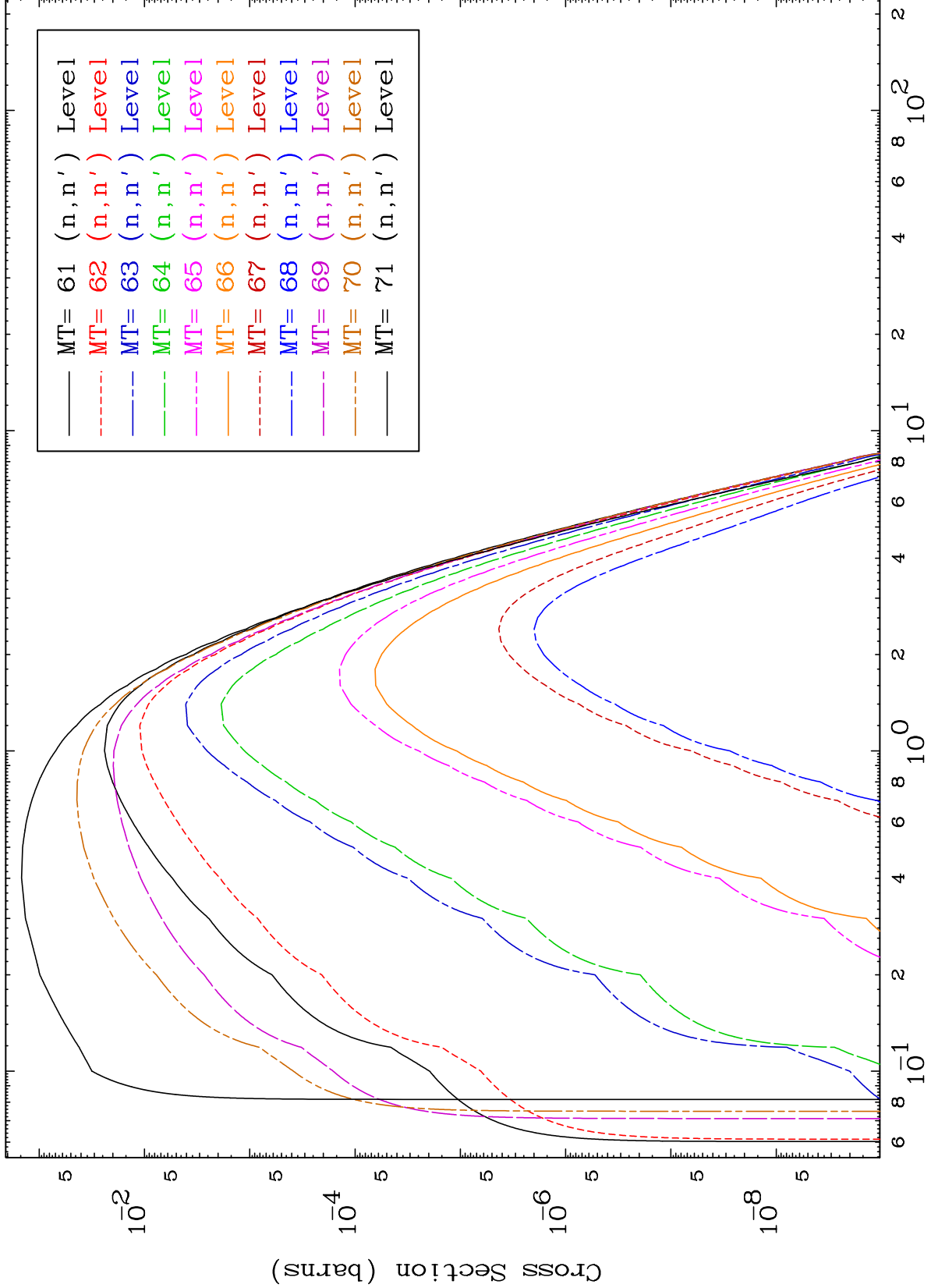


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(n,n') Level

101-Md-249

293 Kelvin Cross Sections



9

Incident Energy (MeV)

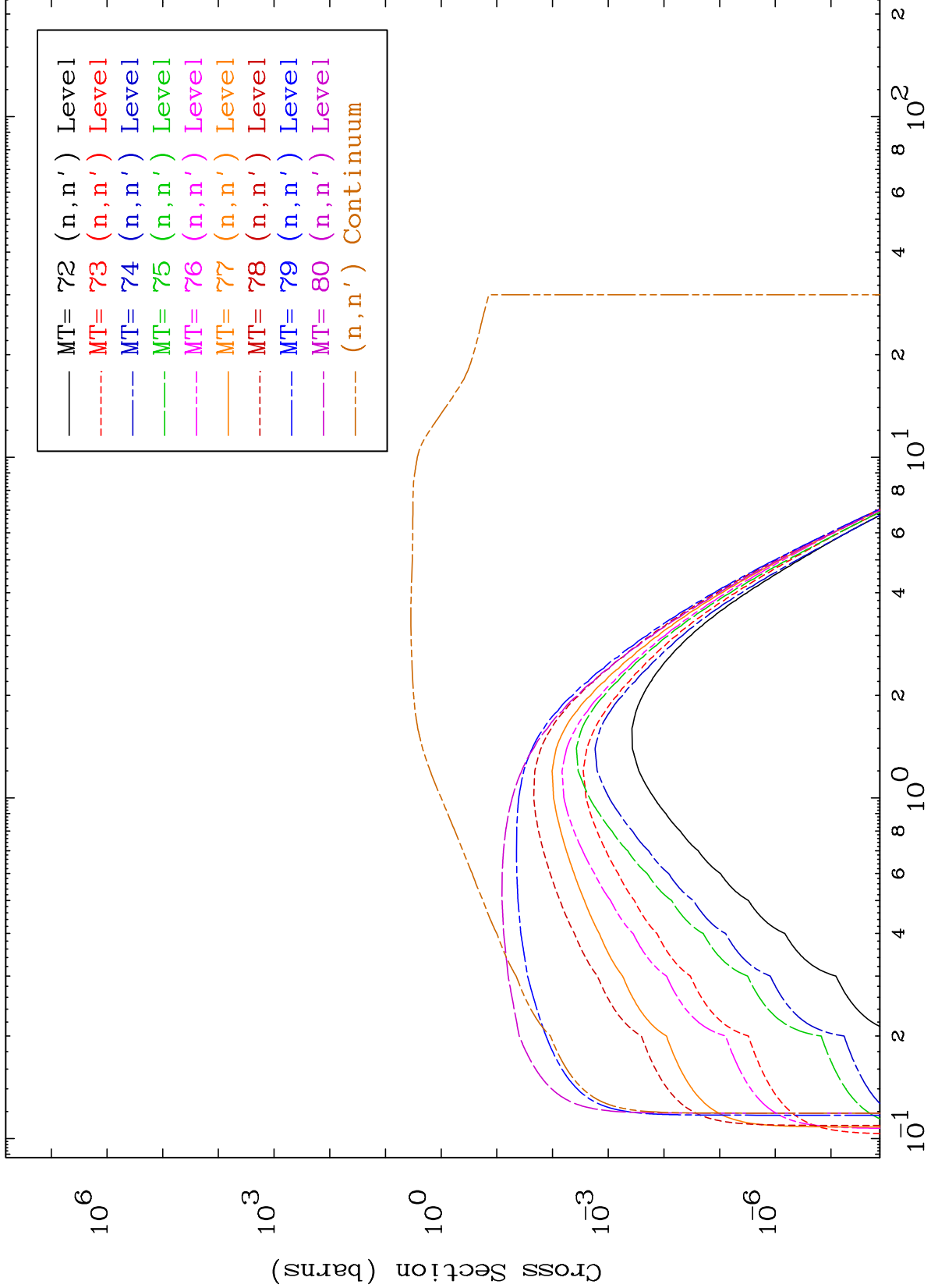
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(n,n') Level

101-Md-249

293 Kelvin Cross Sections



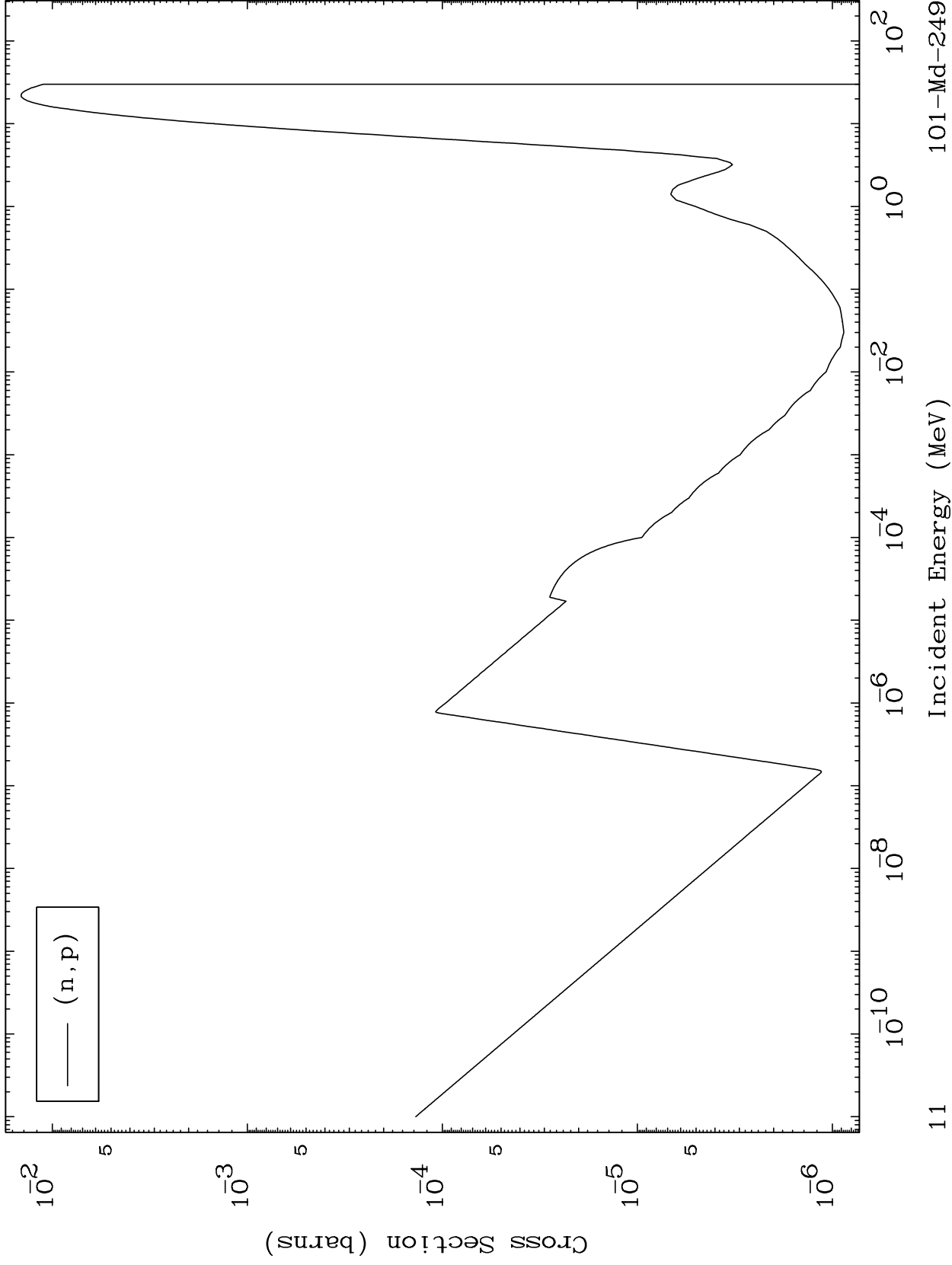
Incident Energy (MeV)

101-Md-249

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(n,p) Levels
293 Kelvin Cross Sections

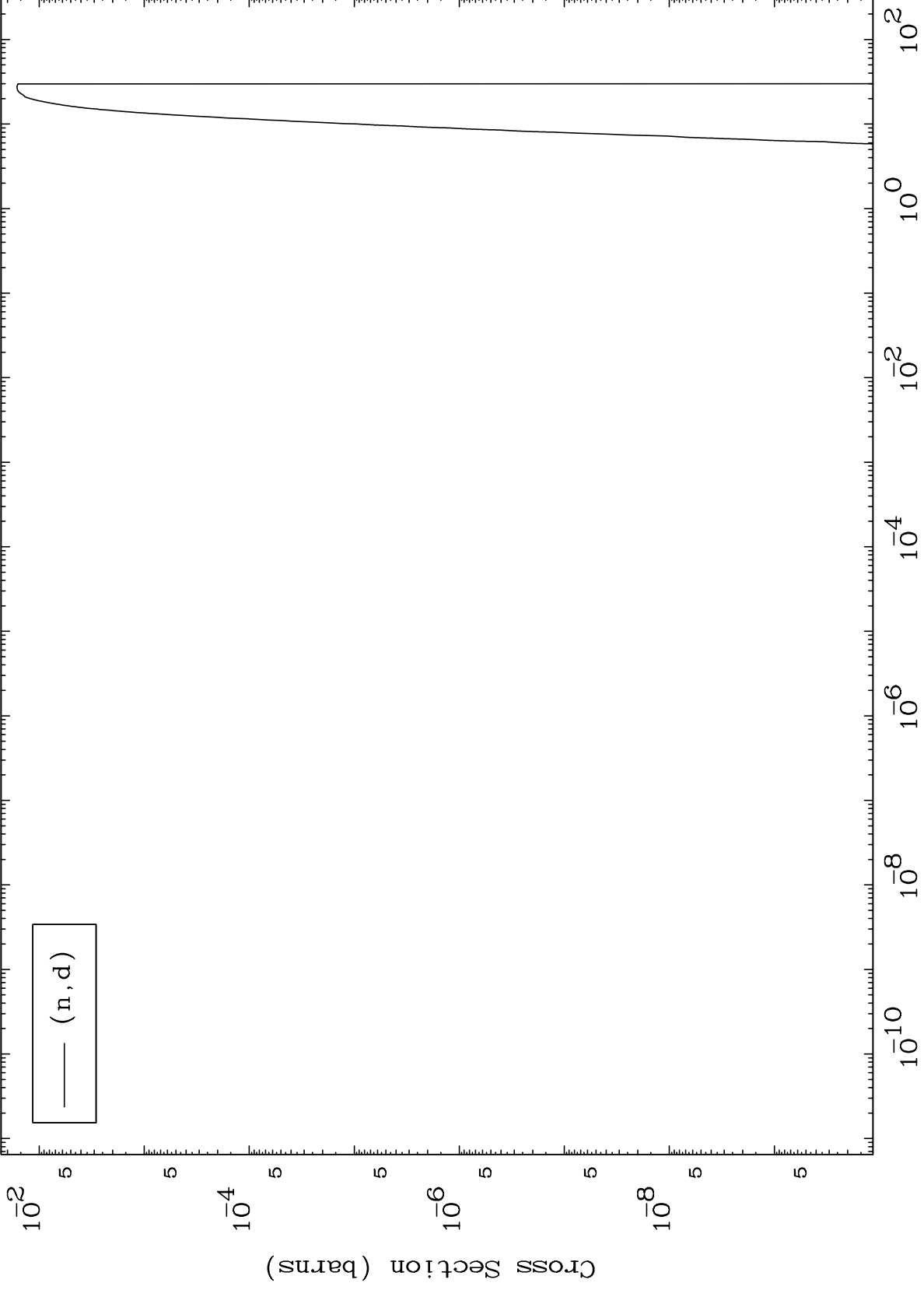
101-Md-249



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-249

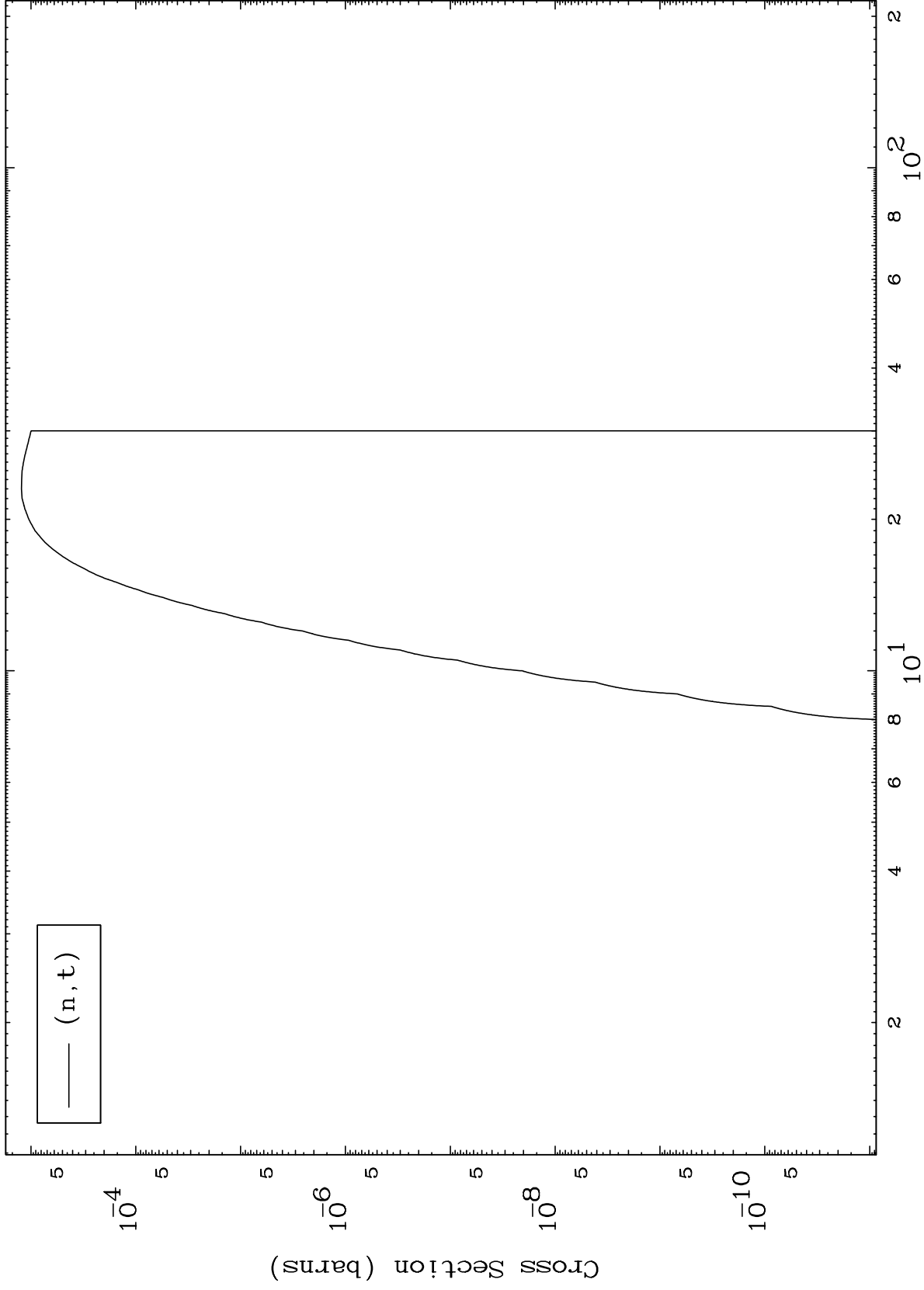


(n,d)

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(n,t) Levels
293 Kelvin Cross Sections

101-Md-249



13

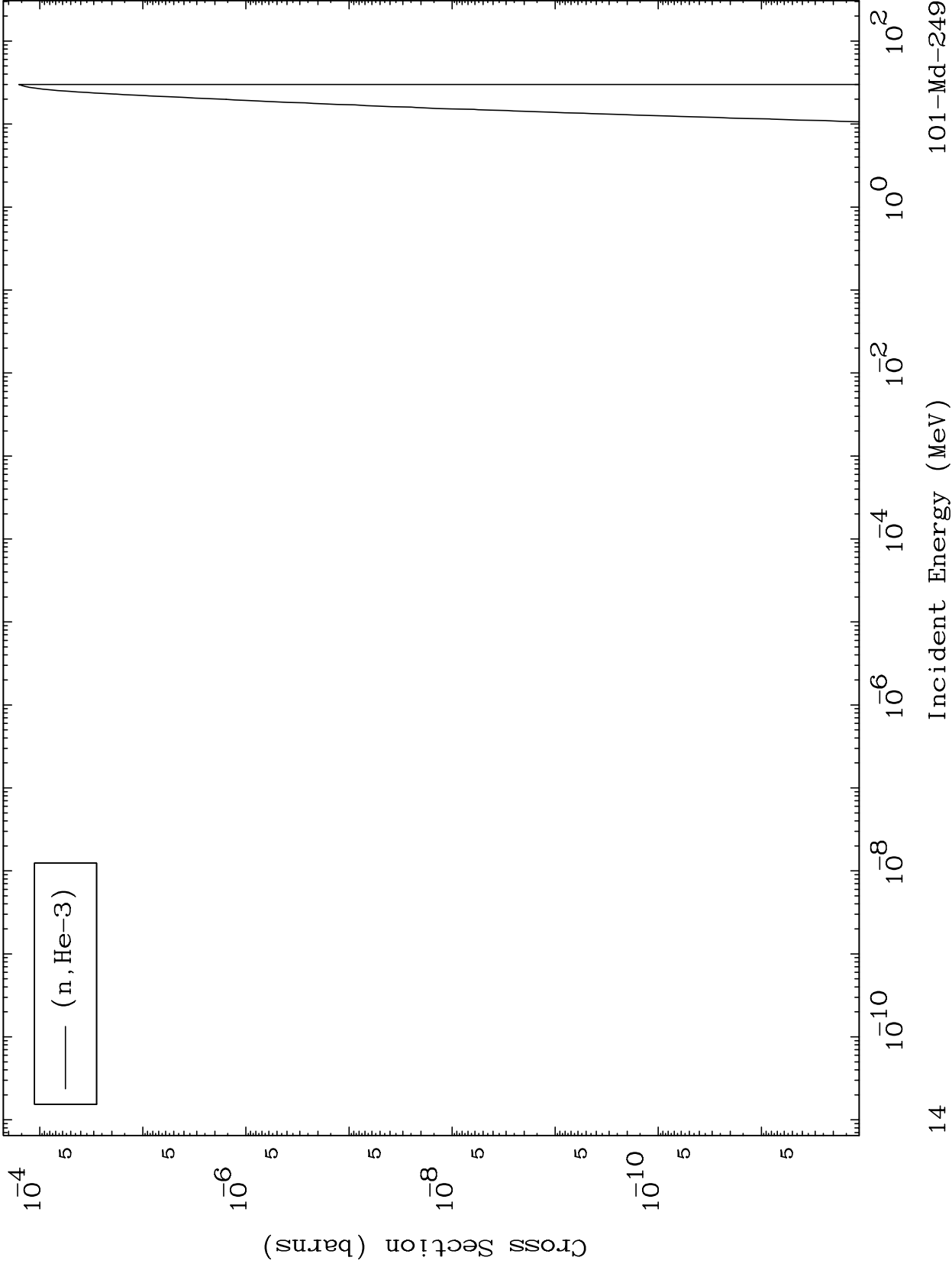
Incident Energy (MeV)

101-Md-249

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(n,He3) Levels
293 Kelvin Cross Sections

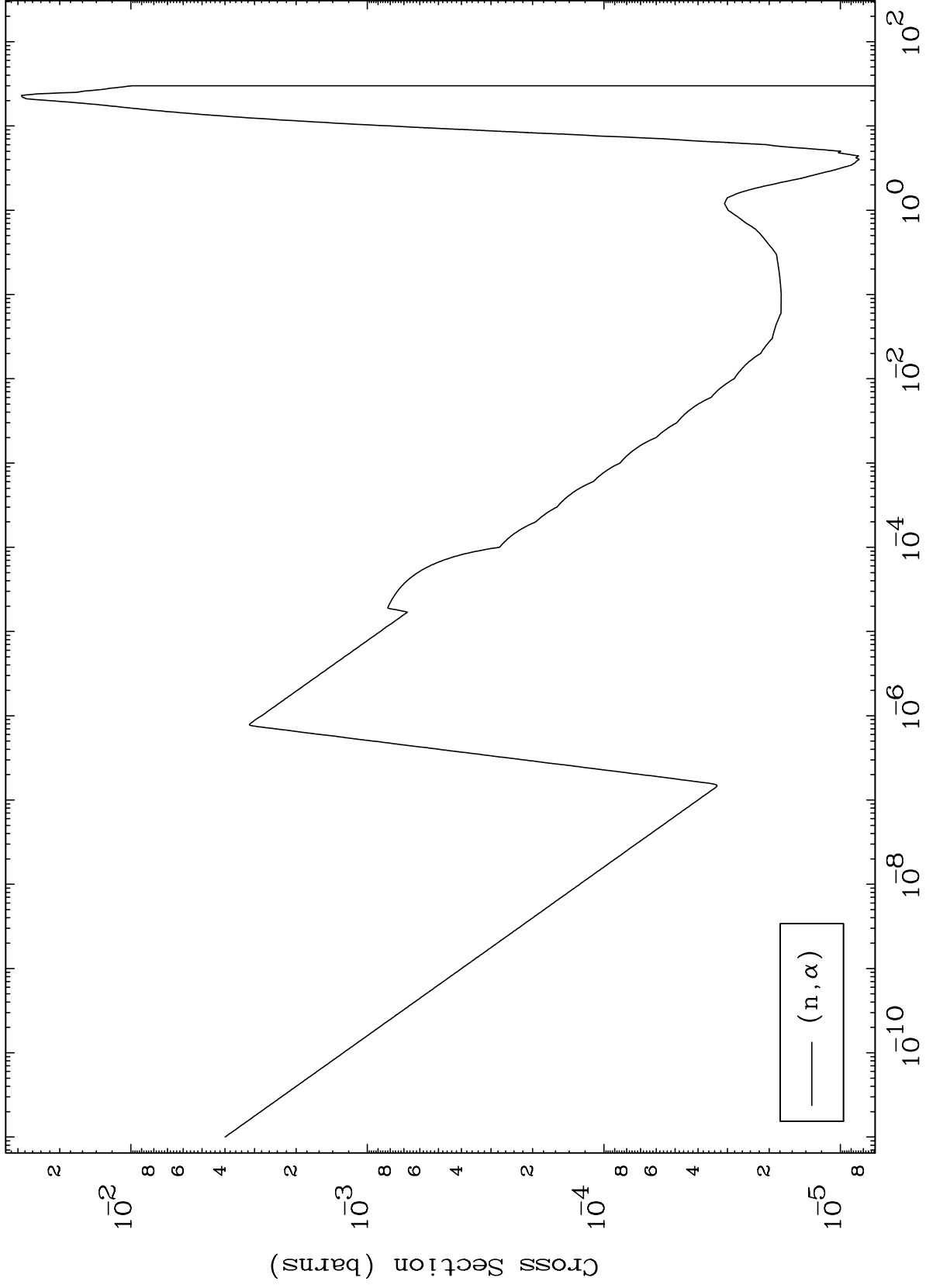
101-Md-249



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(n,α) Levels
293 Kelvin Cross Sections

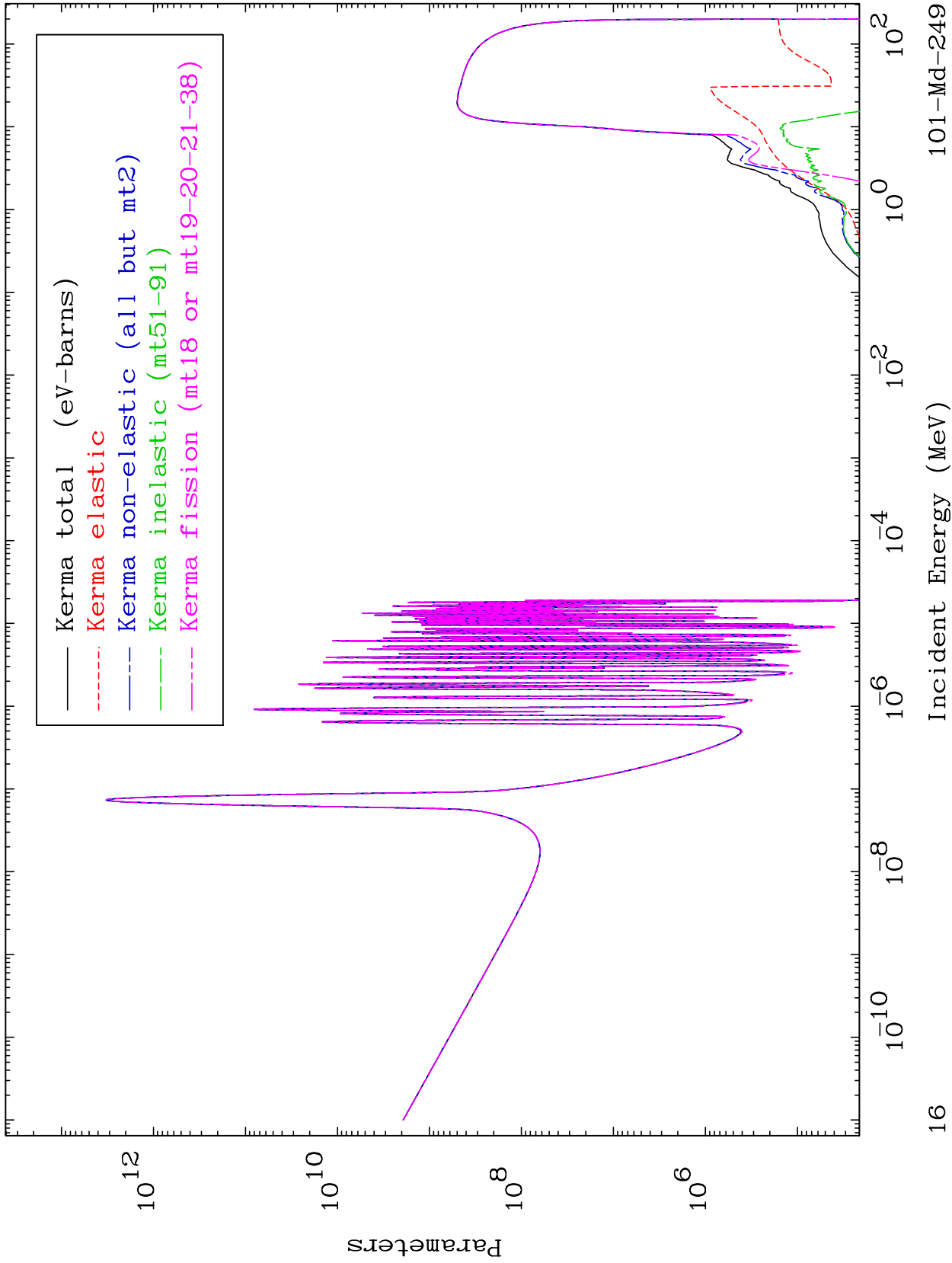
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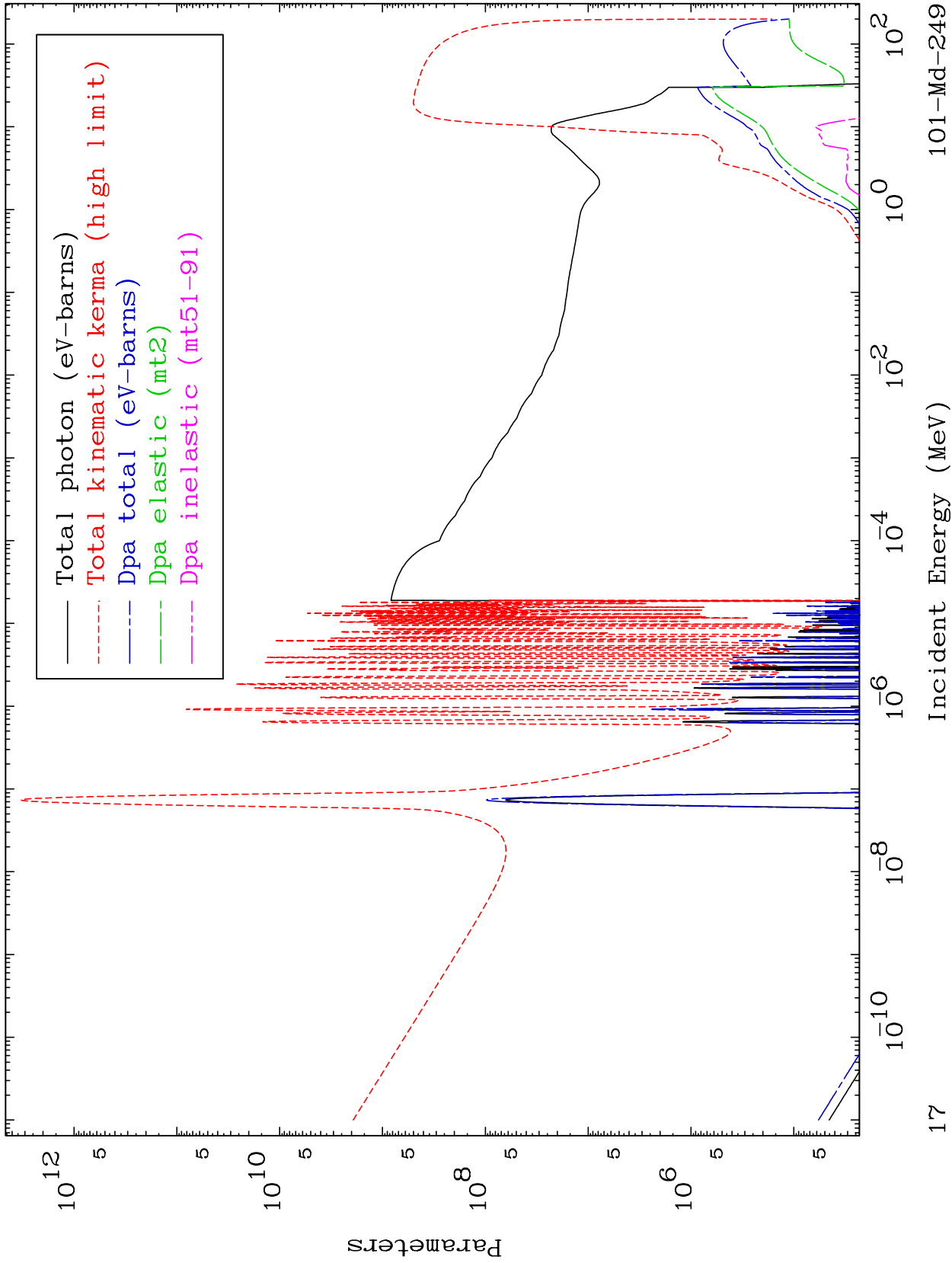


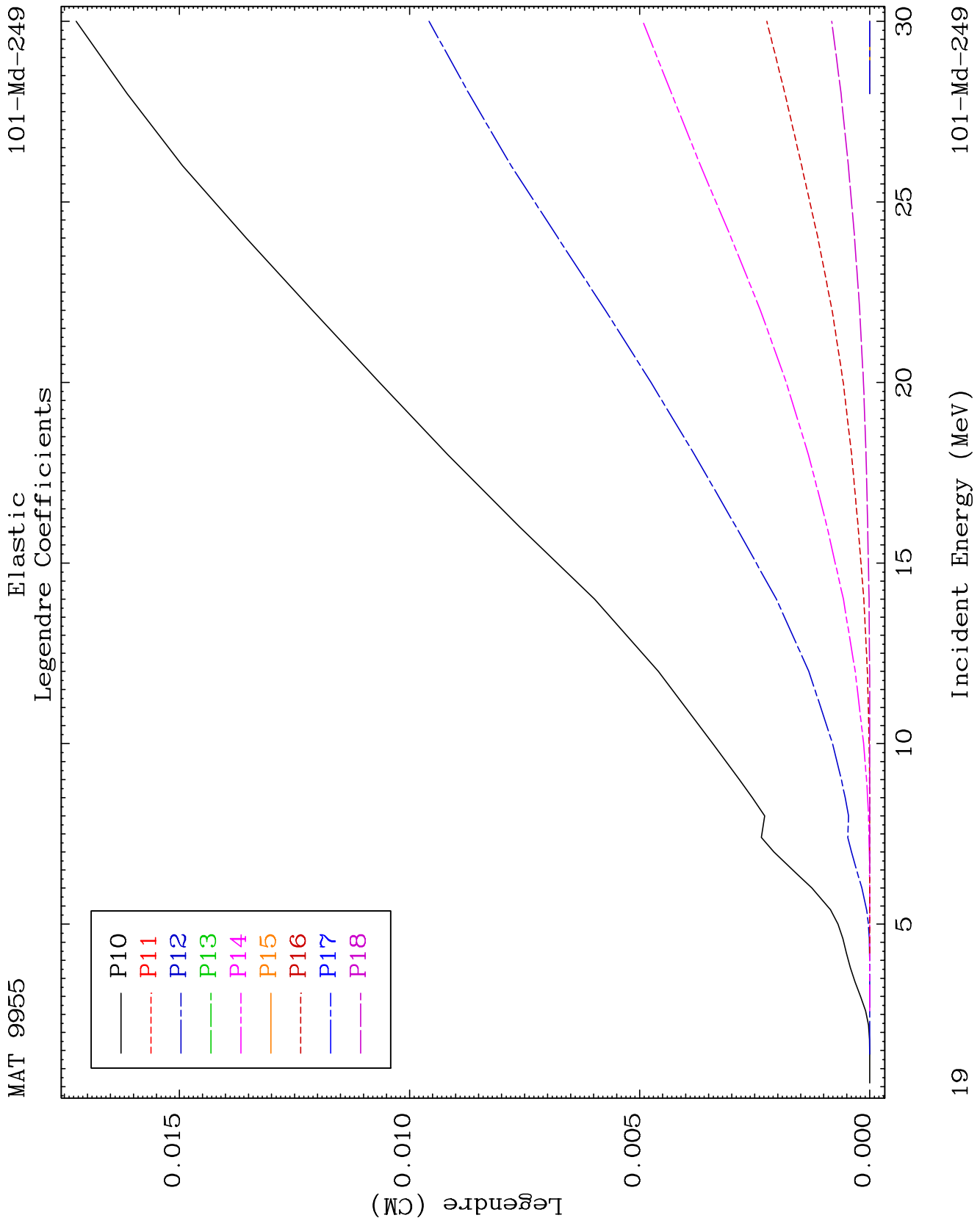
15

Incident Energy (MeV)

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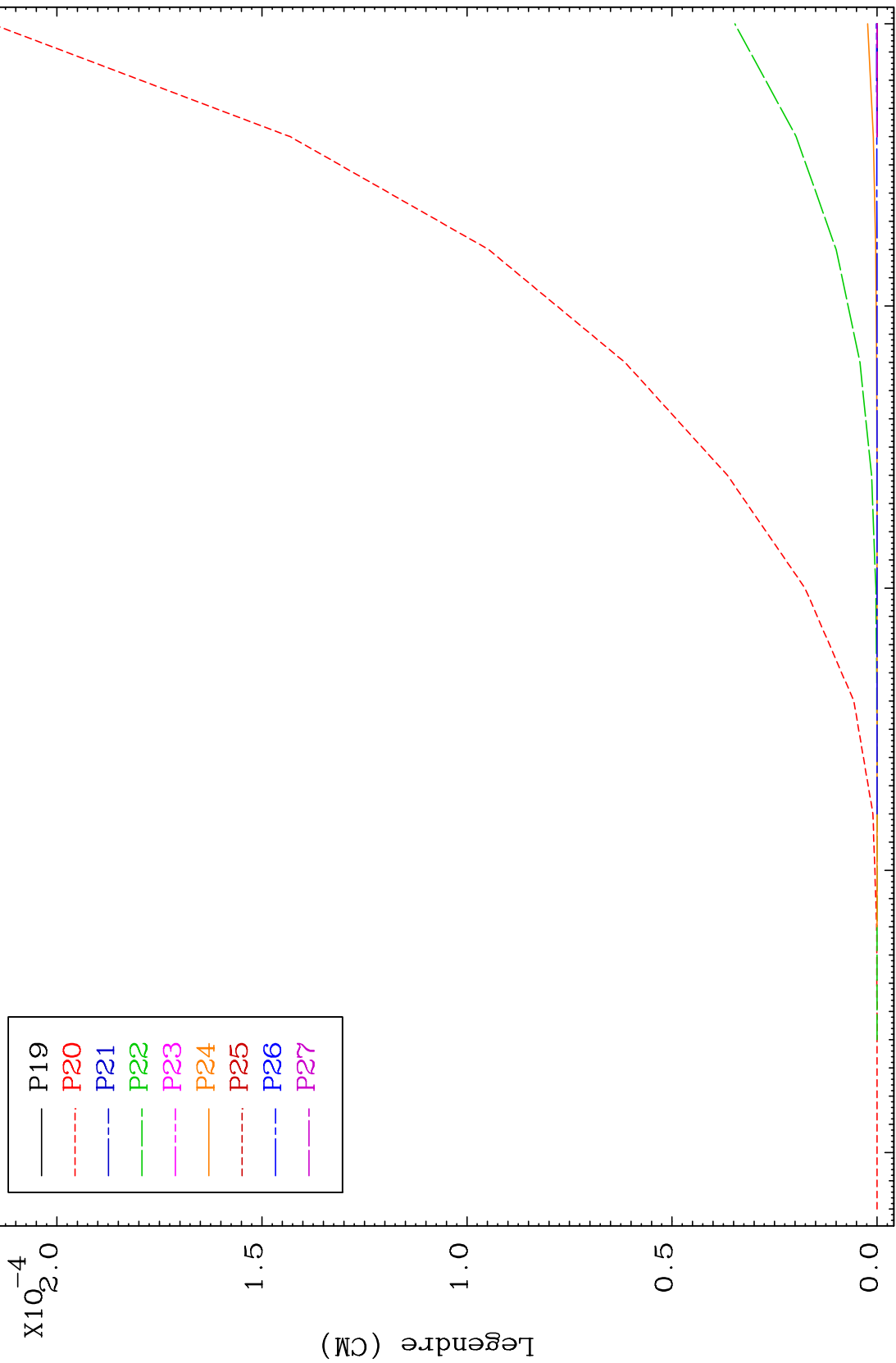
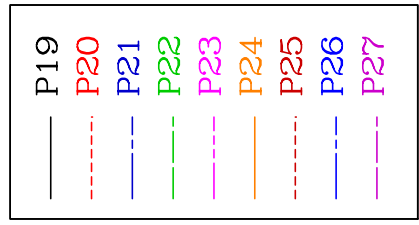




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Elastic Legendre Coefficients

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20

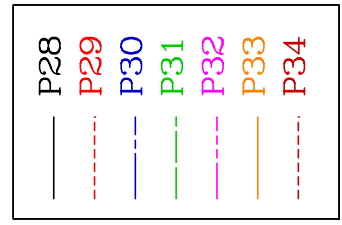
Incident Energy (MeV)

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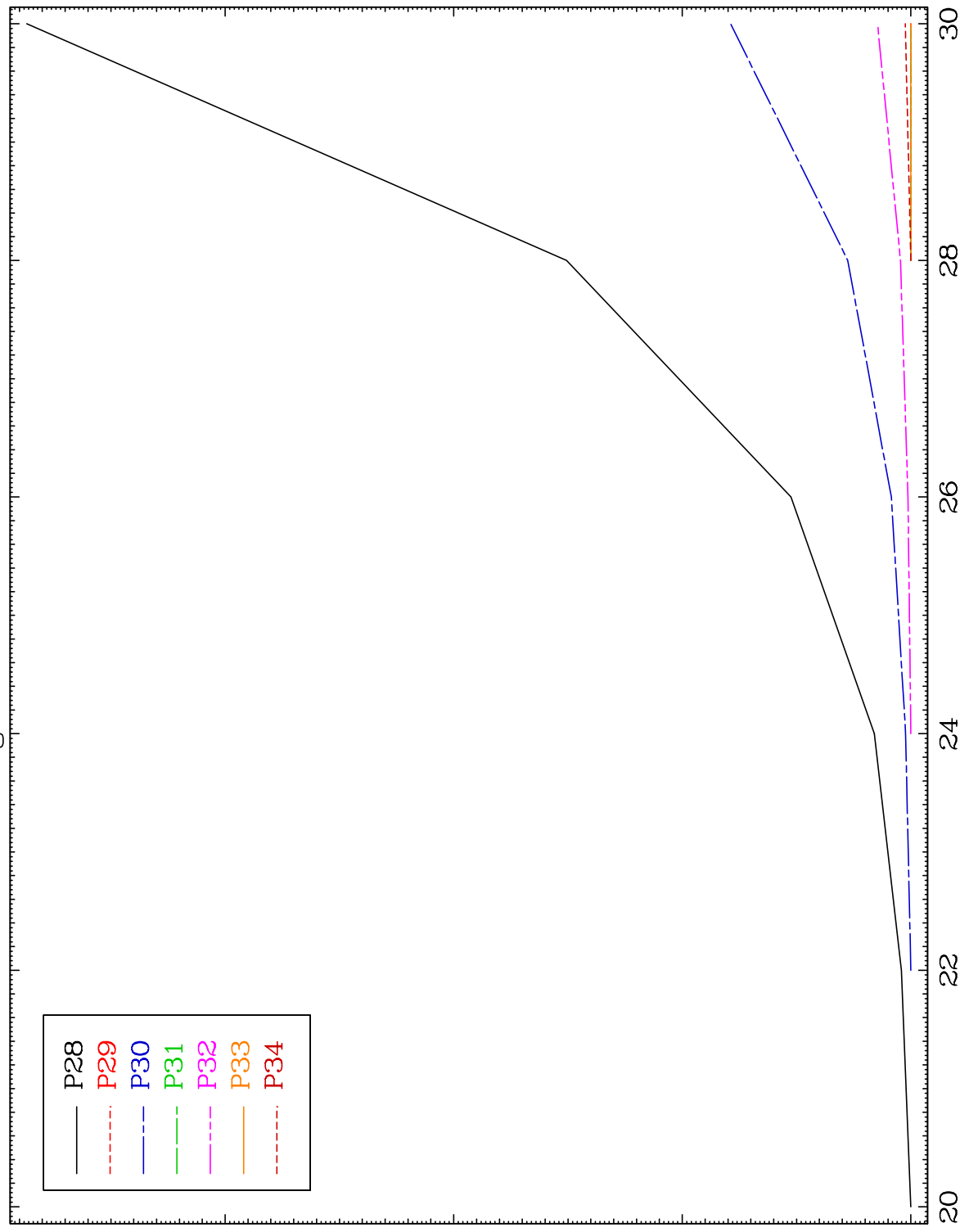
Elastic
Legendre Coefficients

101-Md-249



$\times 10^{-8}$

Legendre (CM)



20

22

24

26

28

30

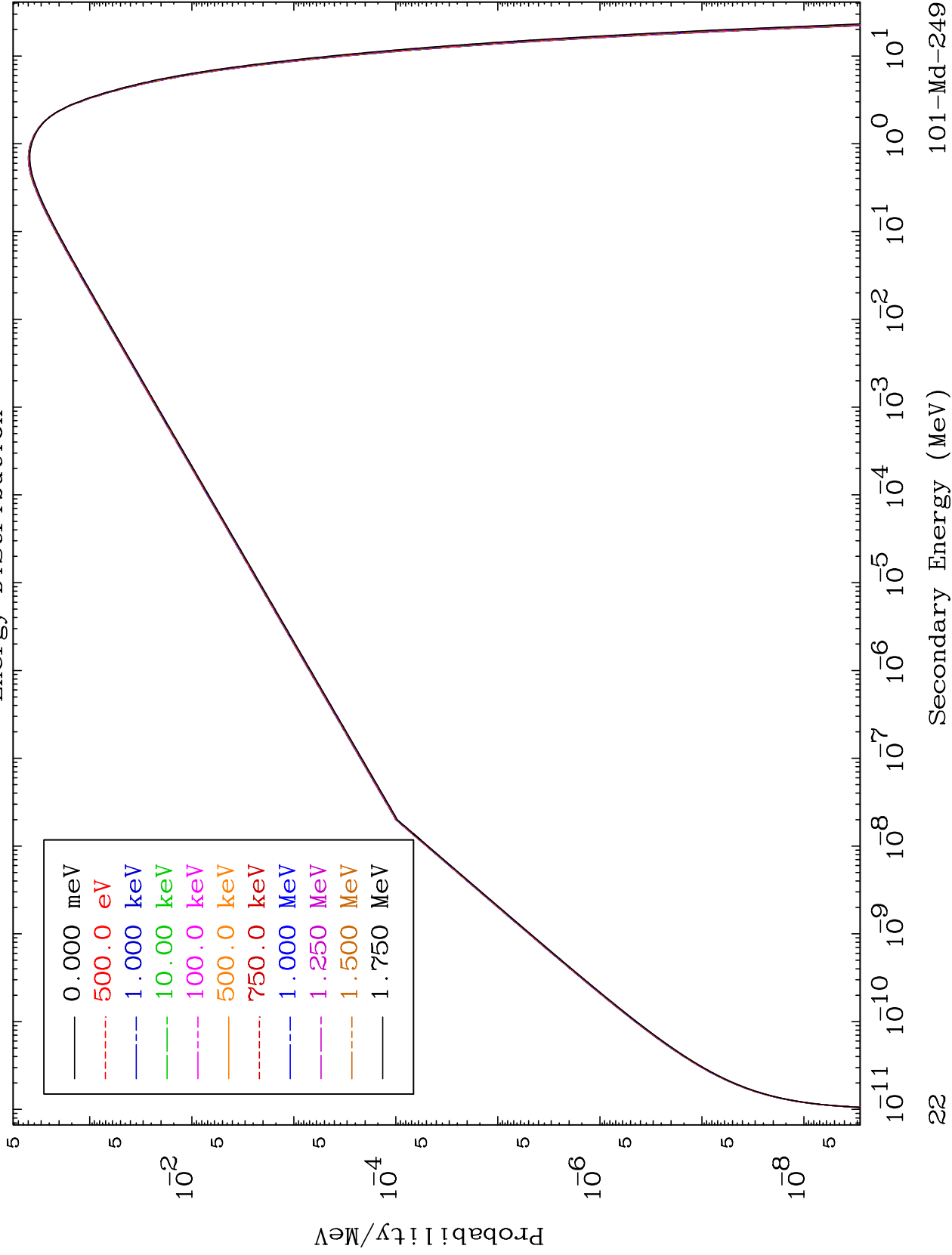
Incident Energy (MeV)

101-Md-249

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Fission Energy Distribution

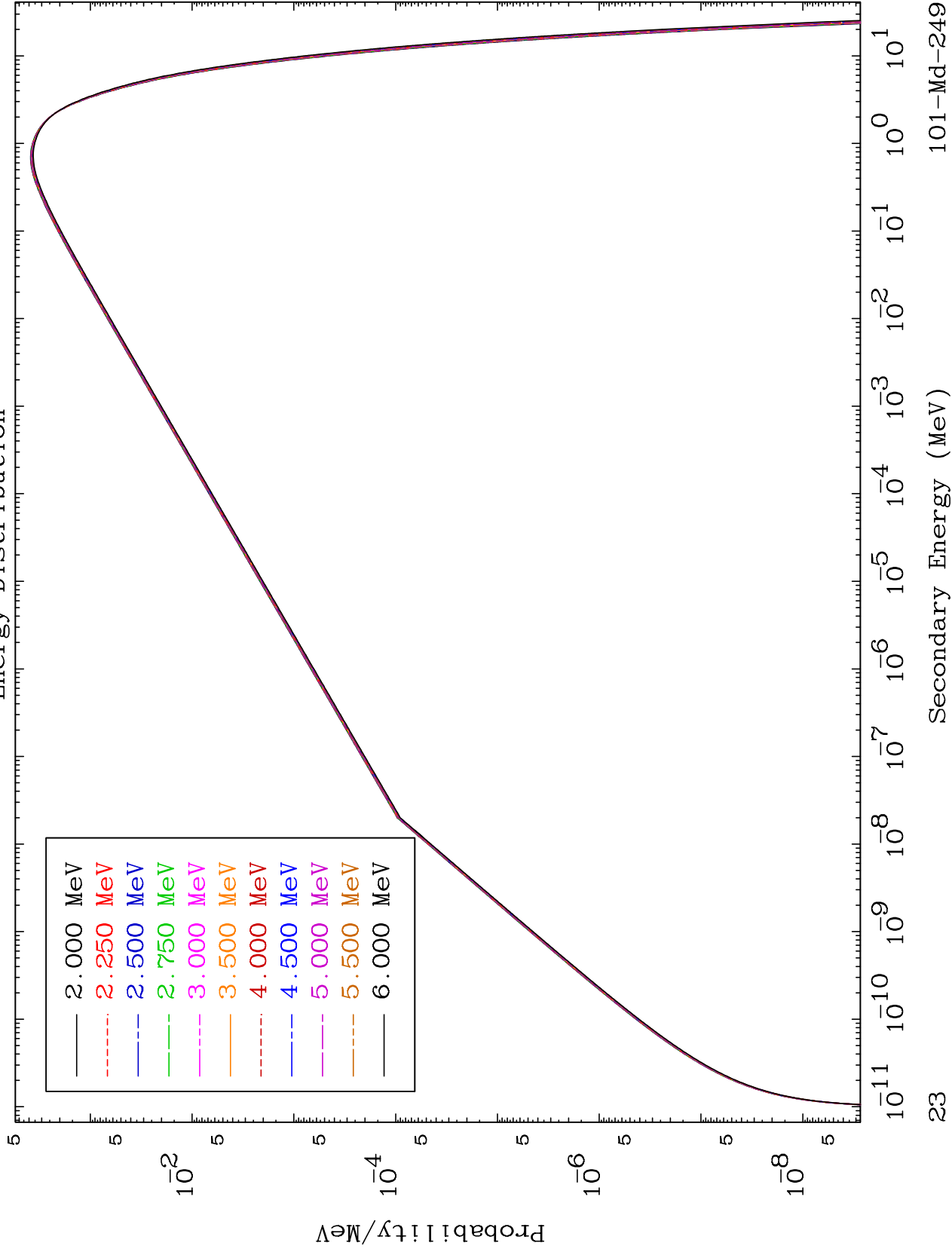
101-Md-249



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Fission
Energy Distribution

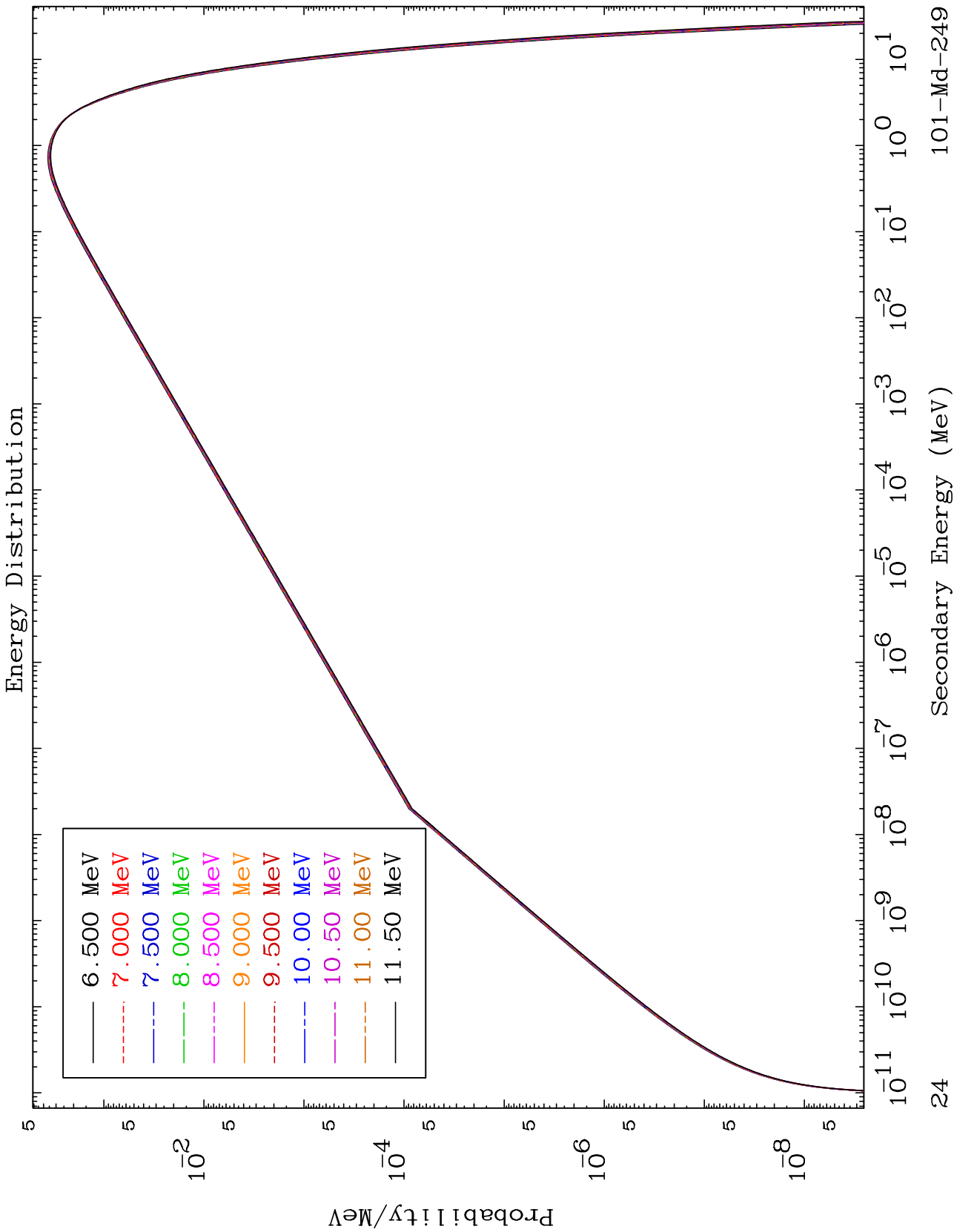
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Fission

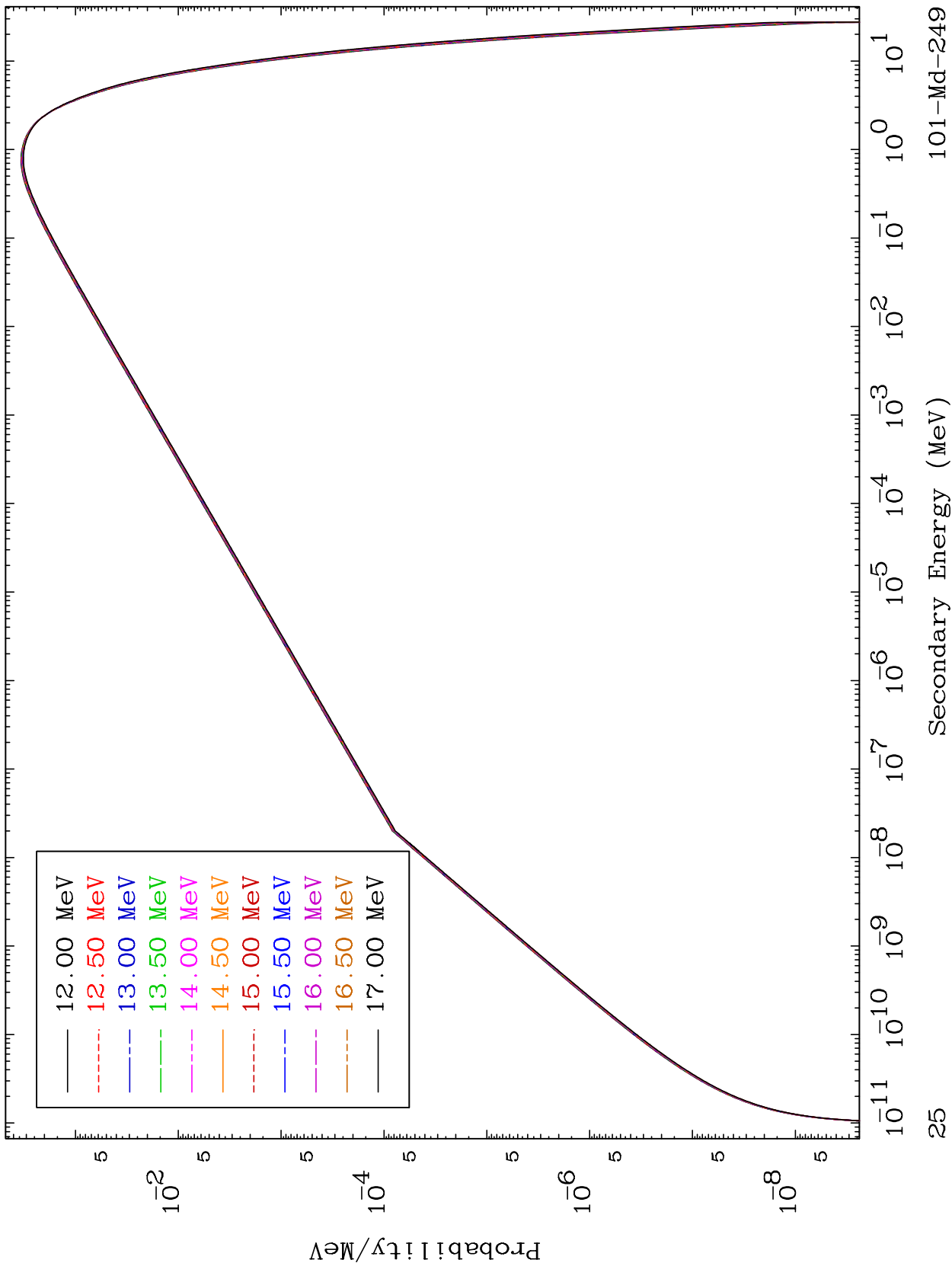
101-Md-249



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Fission
Energy Distribution

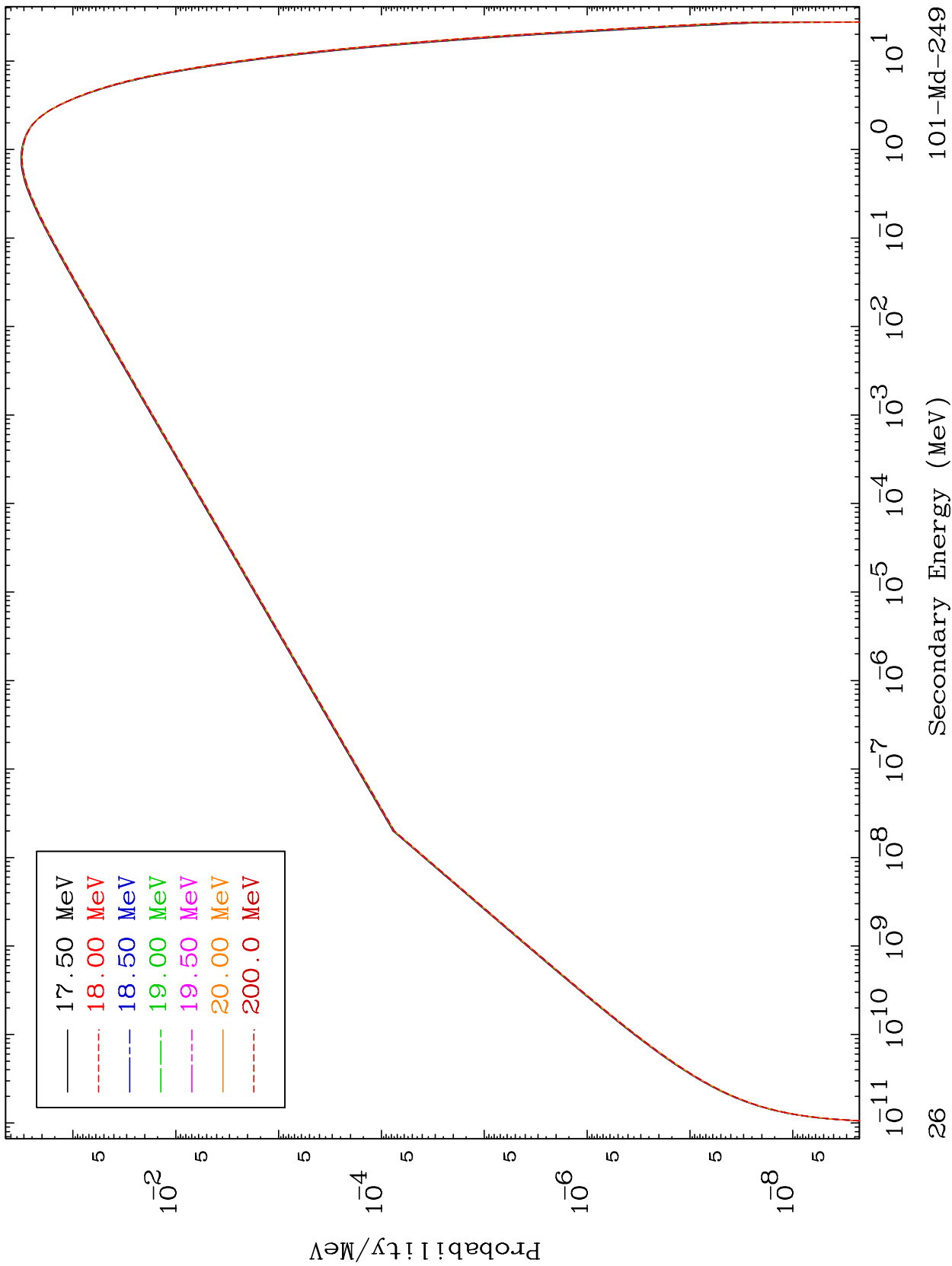
101-Md-249



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Fission Energy Distribution

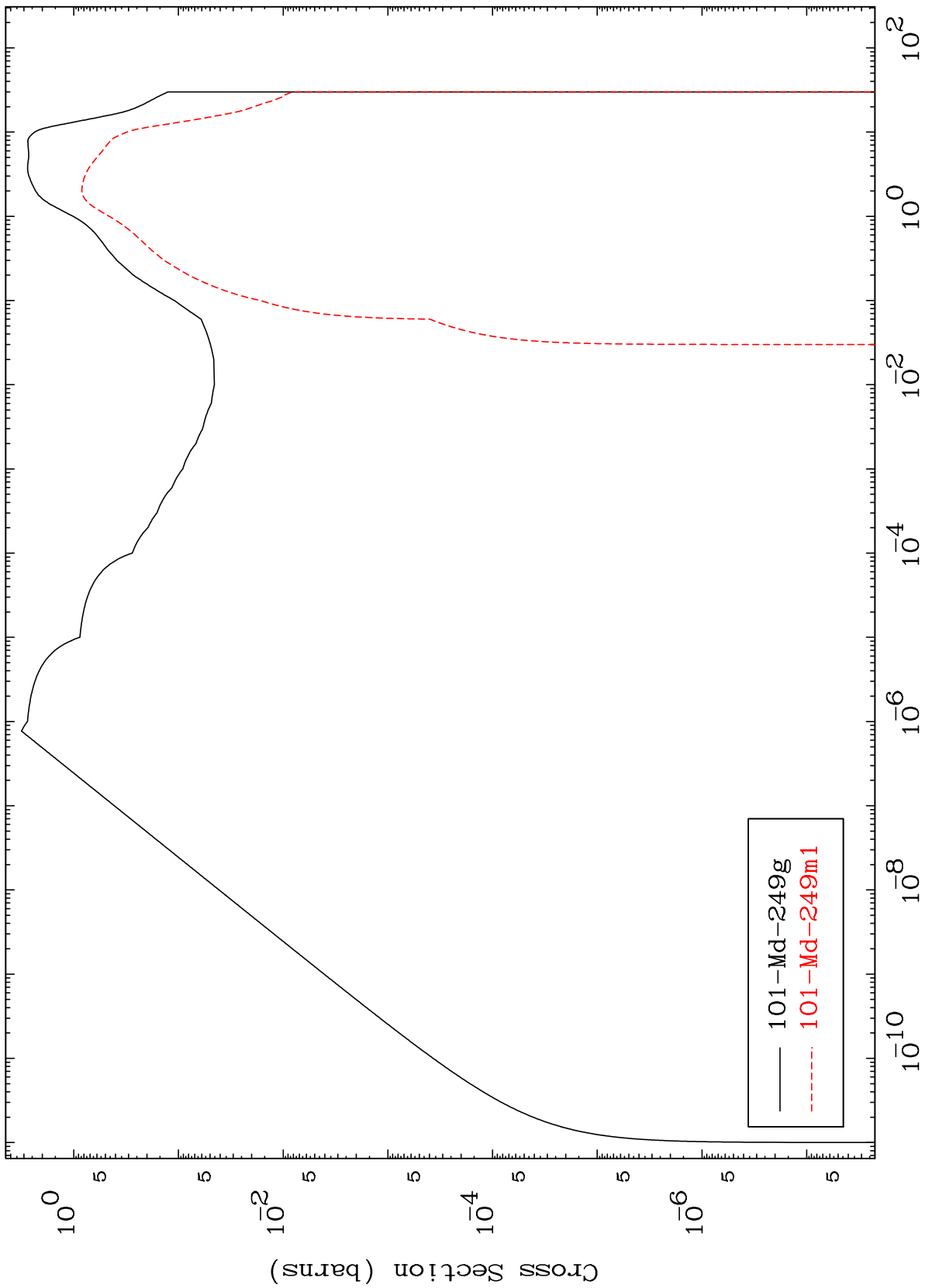
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101-Md-249

Inelastic
Radionuclide Production Cross Section



101-Md-249

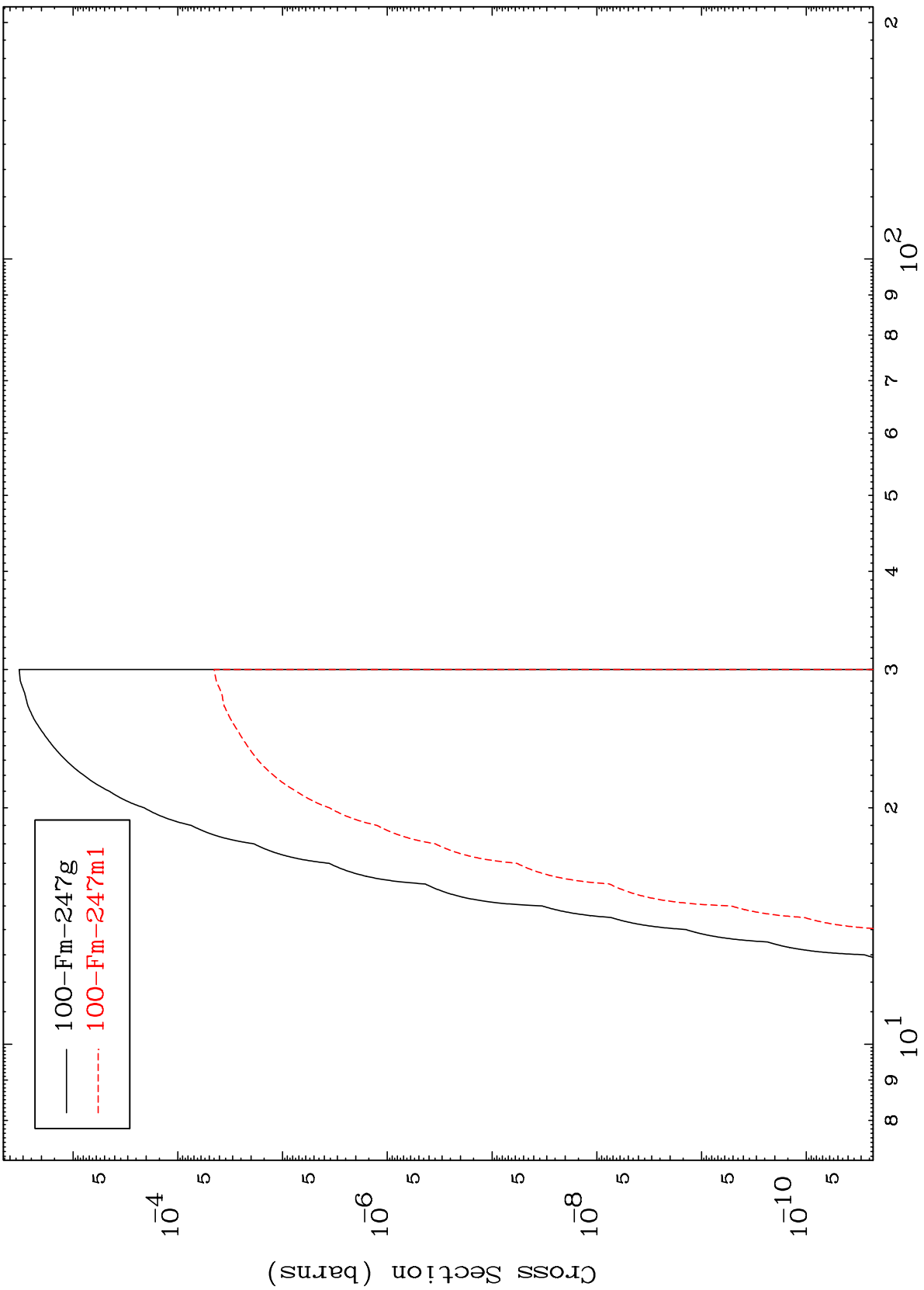
Incident Energy (MeV)

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(n,n') d

101-Md-249

Radionuclide Production Cross Section



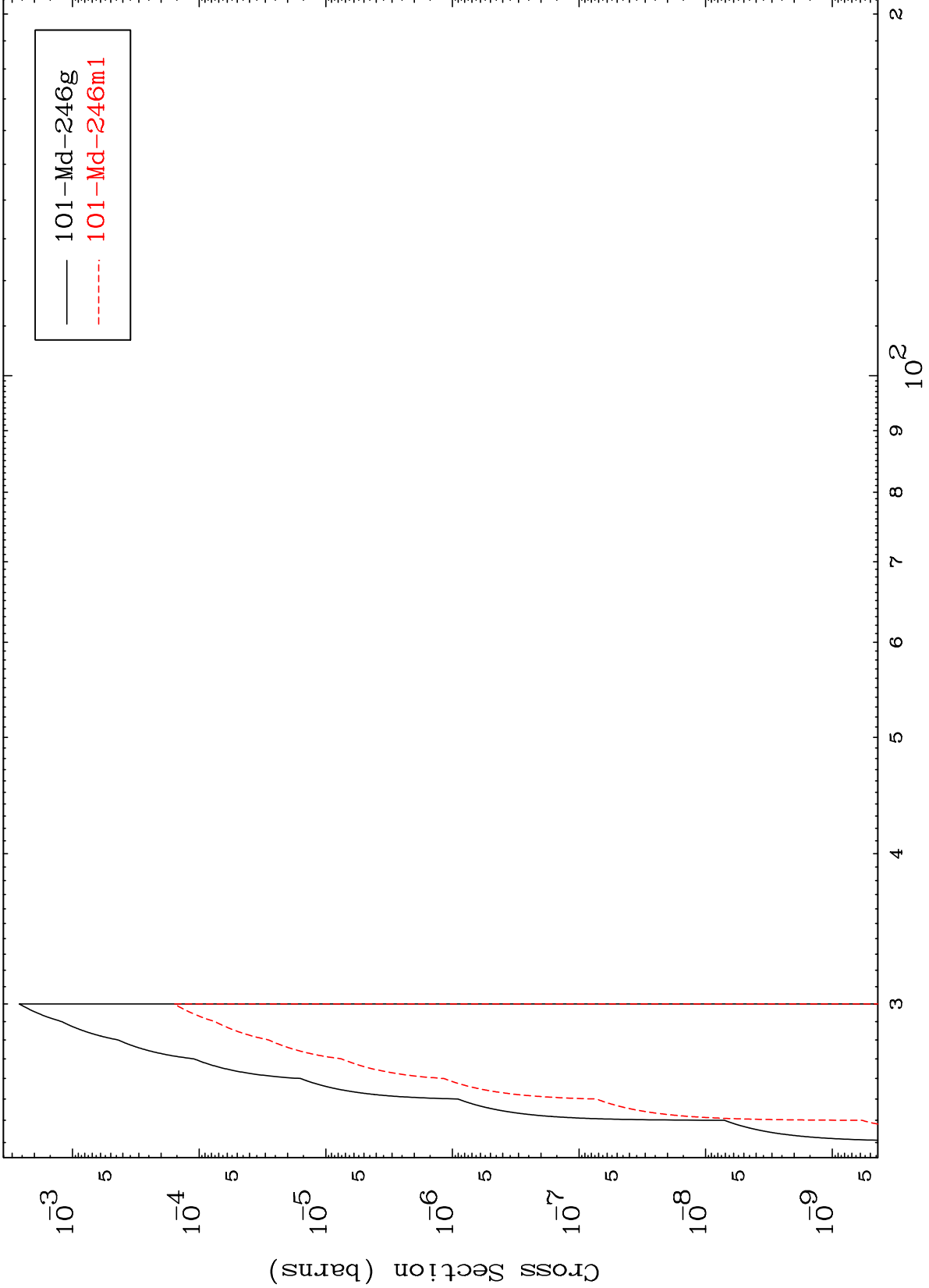
100-Fm-247g
100-Fm-247m1

28

Incident Energy (MeV)

101-Md-249

Radionuclide Production Cross Section

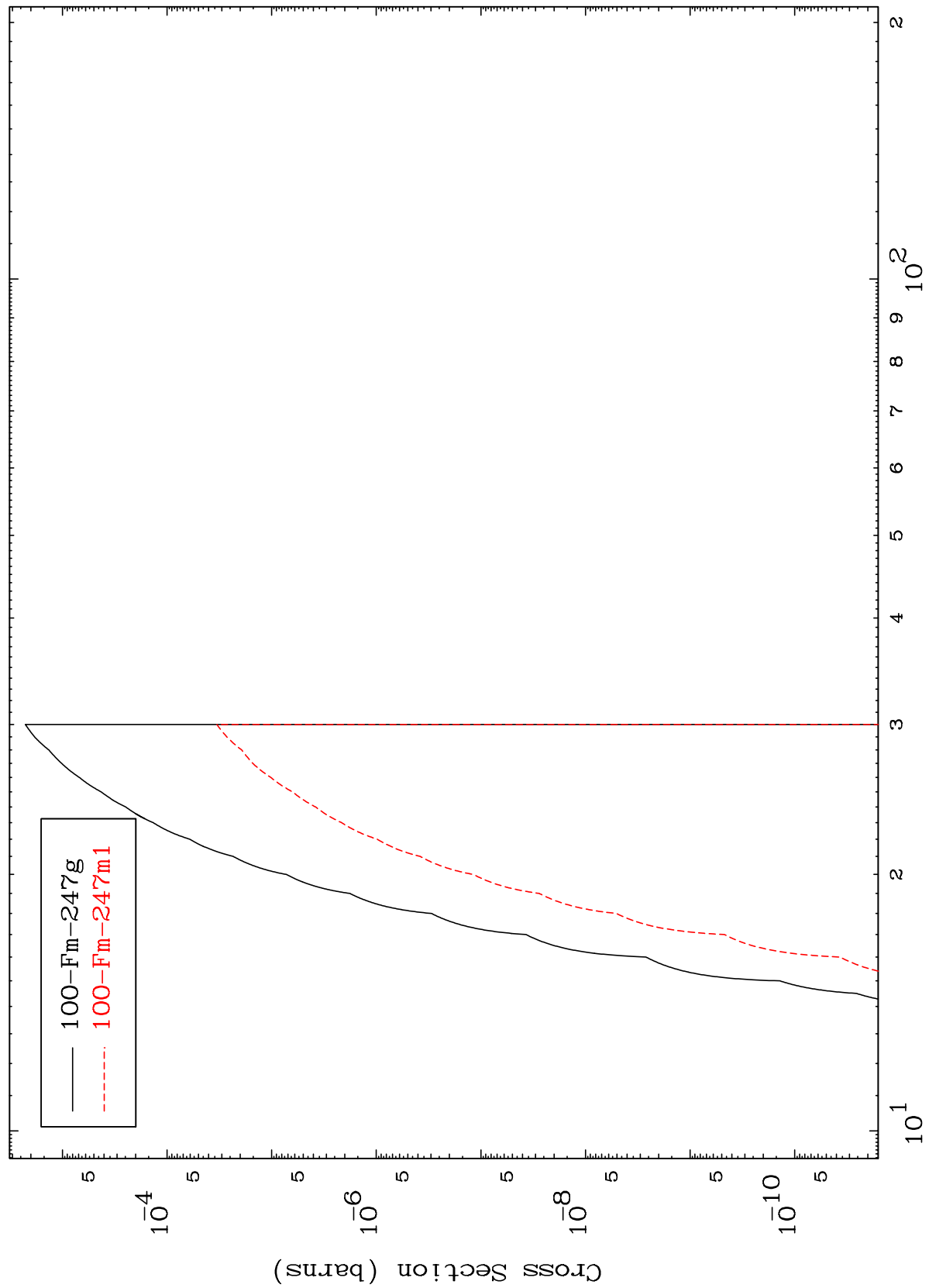


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(n,2n) p

101-Md-249

Radionuclide Production Cross Section



100-Fm-247g
100-Fm-247m1

Incident Energy (MeV)

101-Md-249

30

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101-Md-249

(n, t)
Radionuclide Production Cross Section

