

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

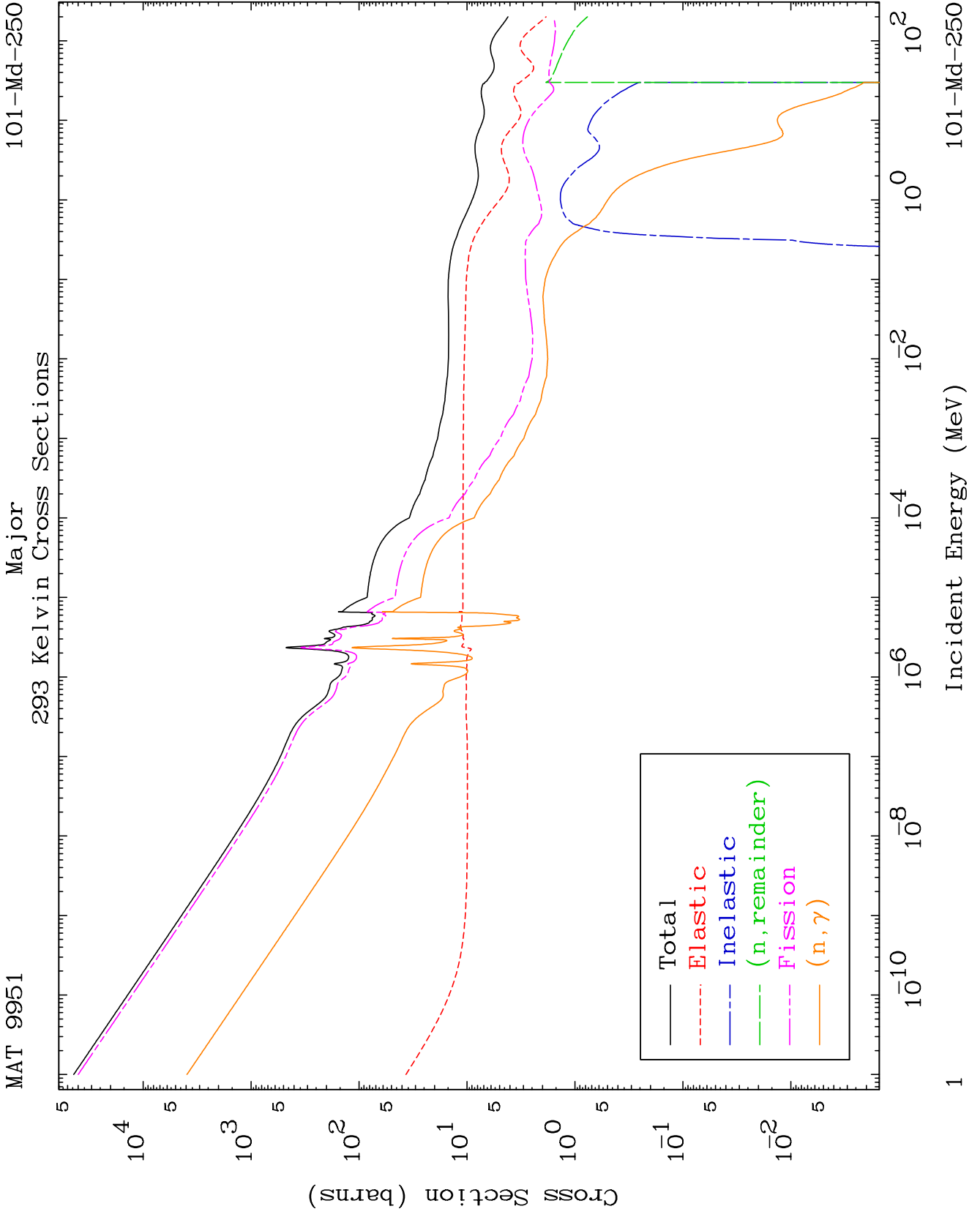
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Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

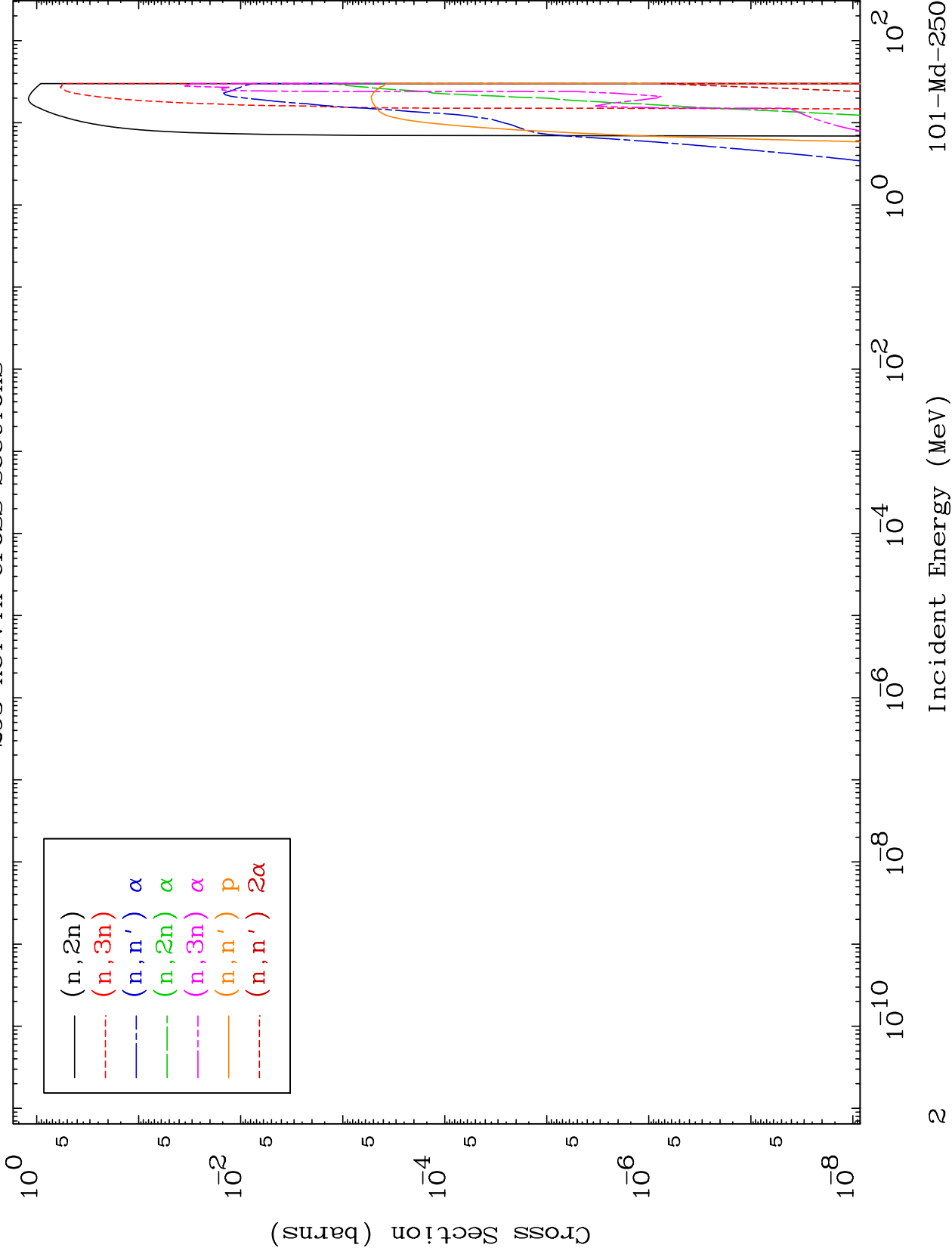
Press Mouse Button to Start



MAT 9951

Neutron Production  
293 Kelvin Cross Sections

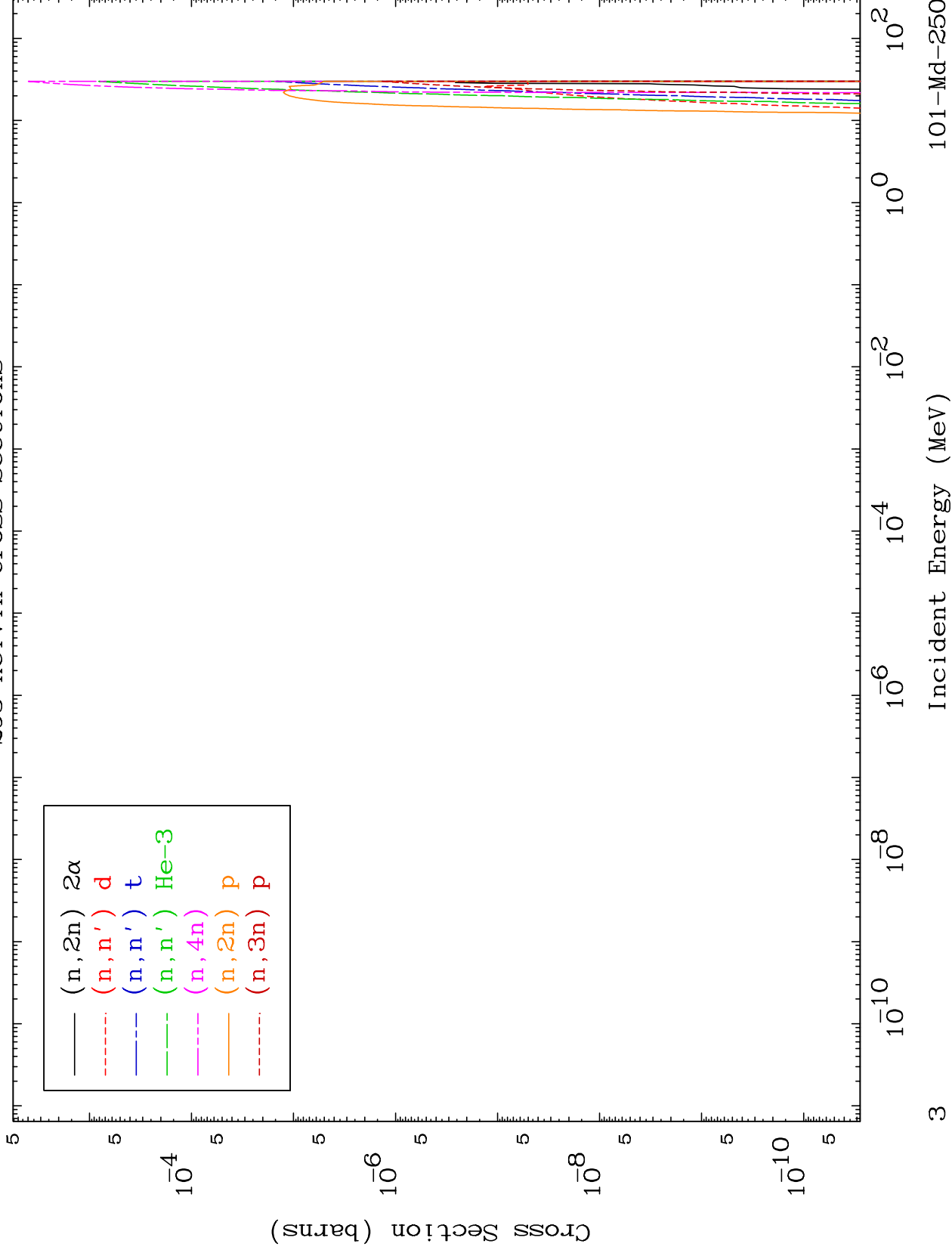
101-Md-250



MAT 9951

Neutron Production  
293 Kelvin Cross Sections

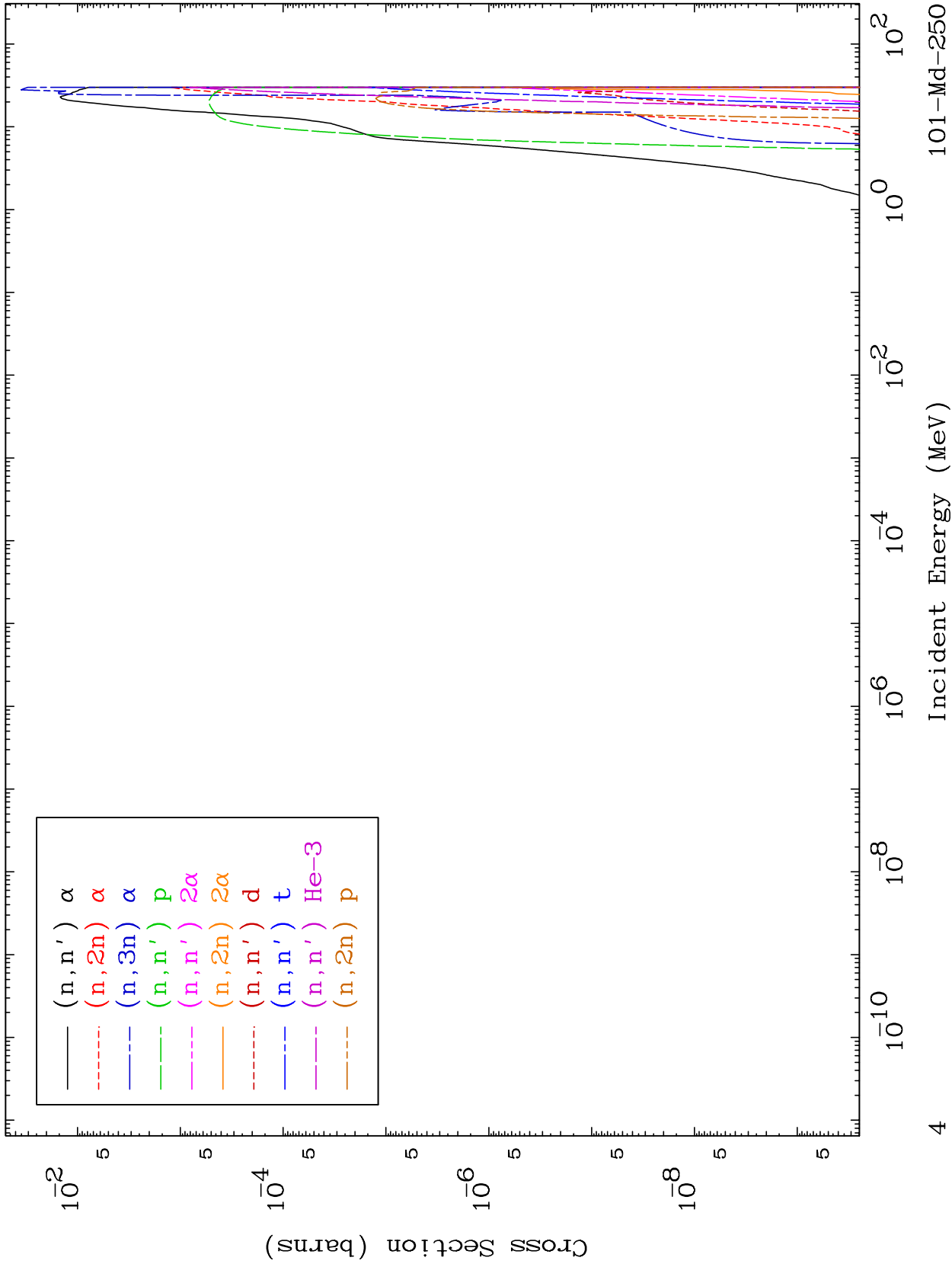
101-Md-250



MAT 9951

Charged Particle  
293 Kelvin Cross Sections

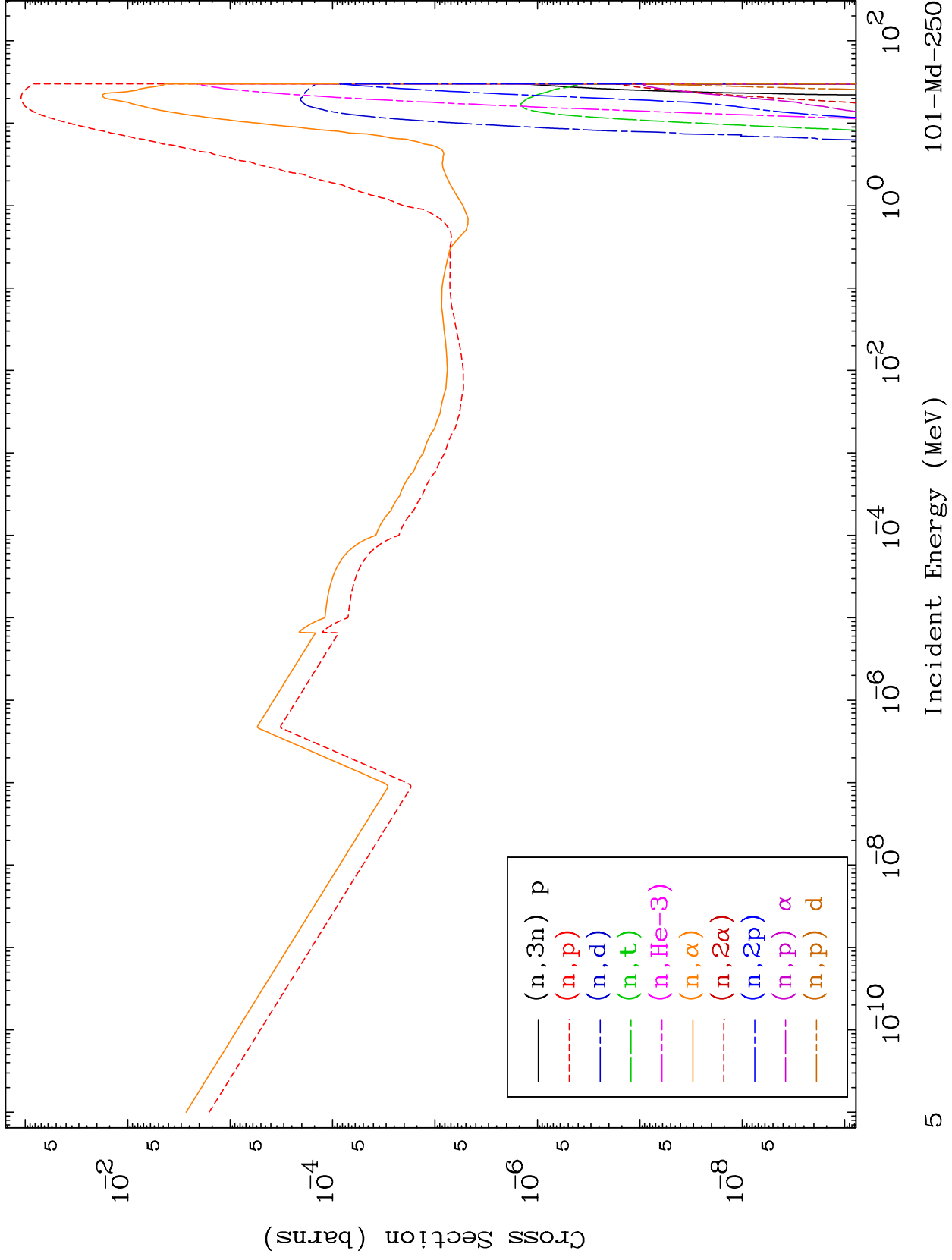
101-Md-250



MAT 9951

Charged Particle  
293 Kelvin Cross Sections

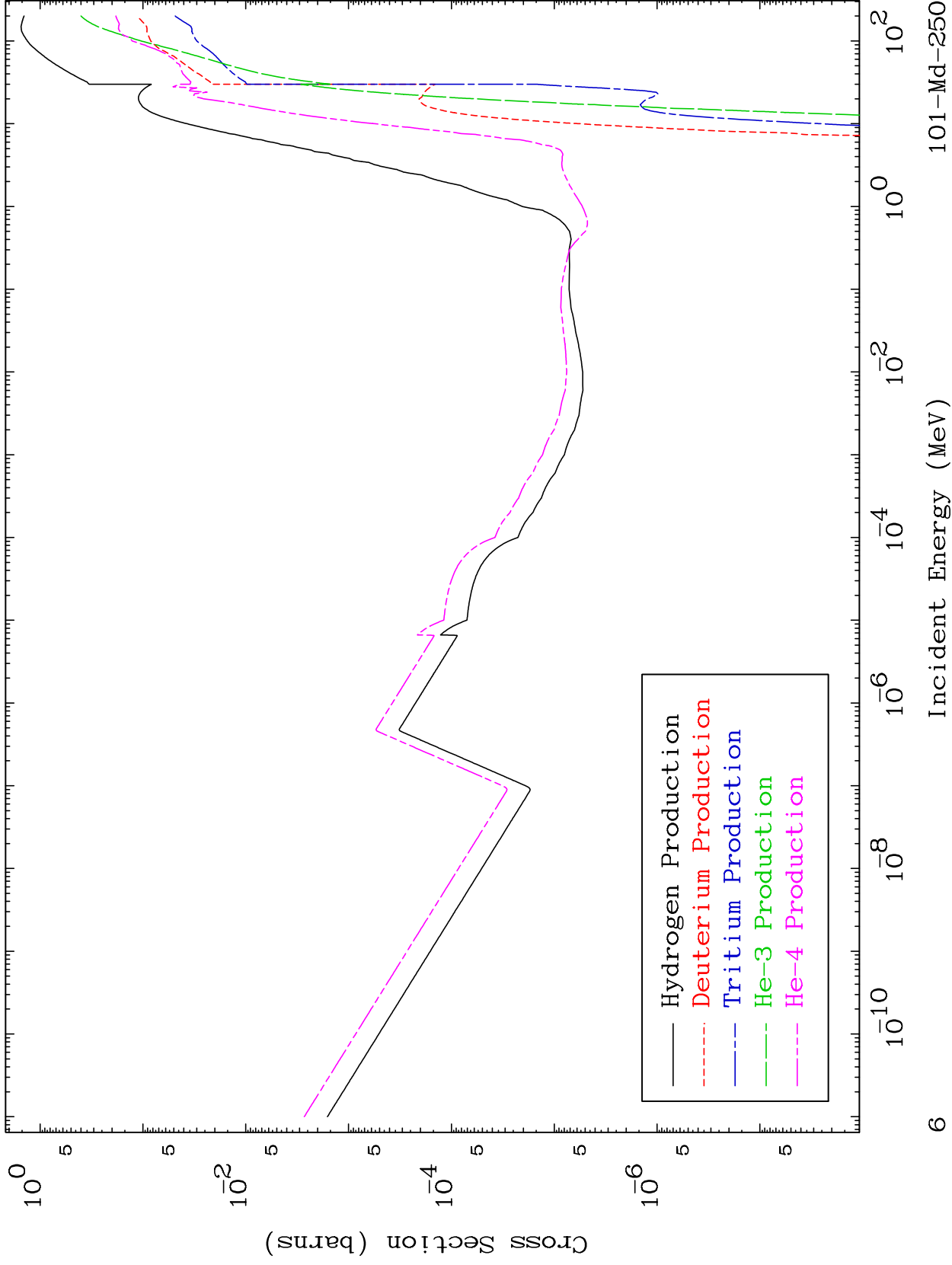
101-Md-250



MAT 9951

Particle Production  
293 Kelvin Cross Sections

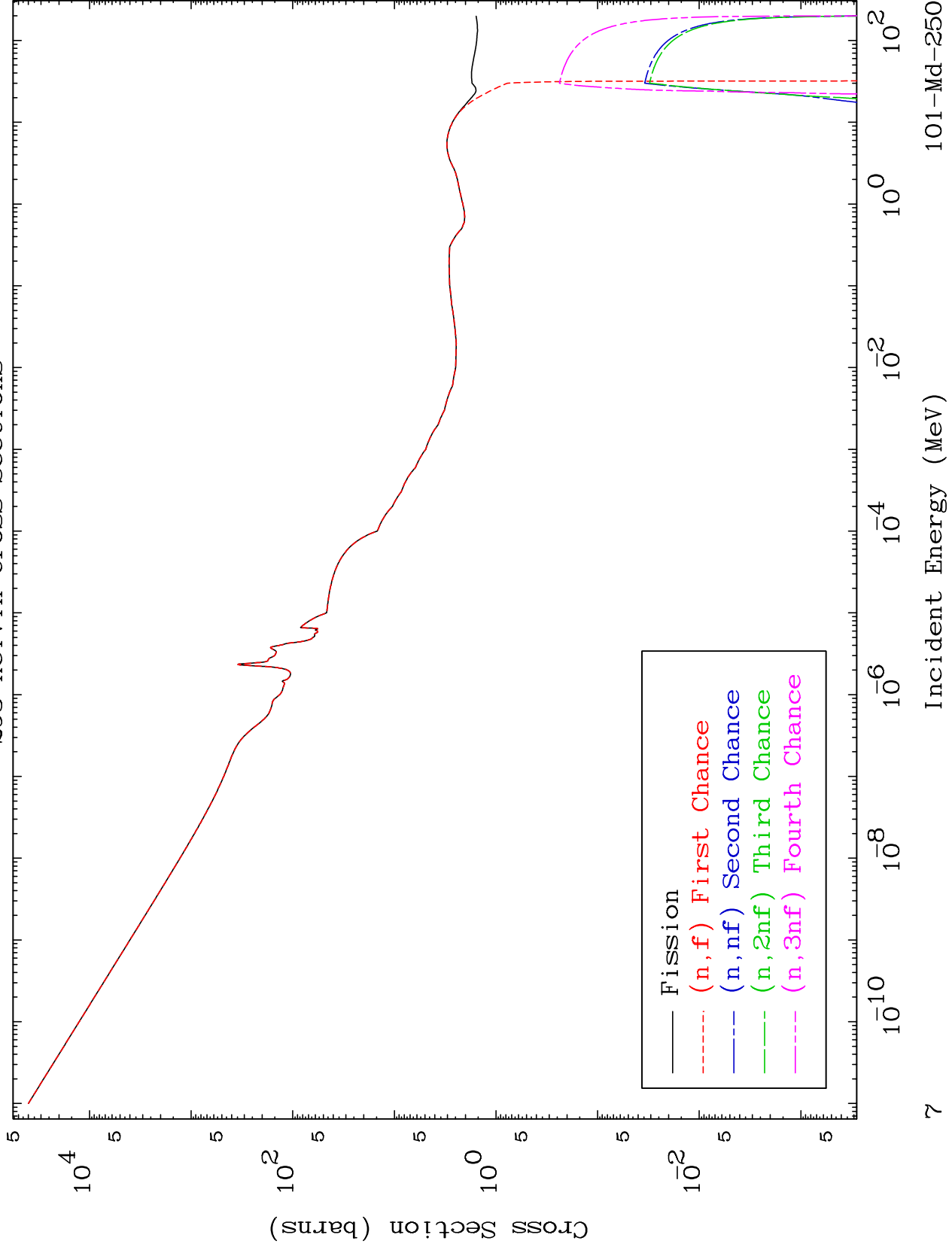
101-Md-250



MAT 9951

Fission  
293 Kelvin Cross Sections

101-Md-250



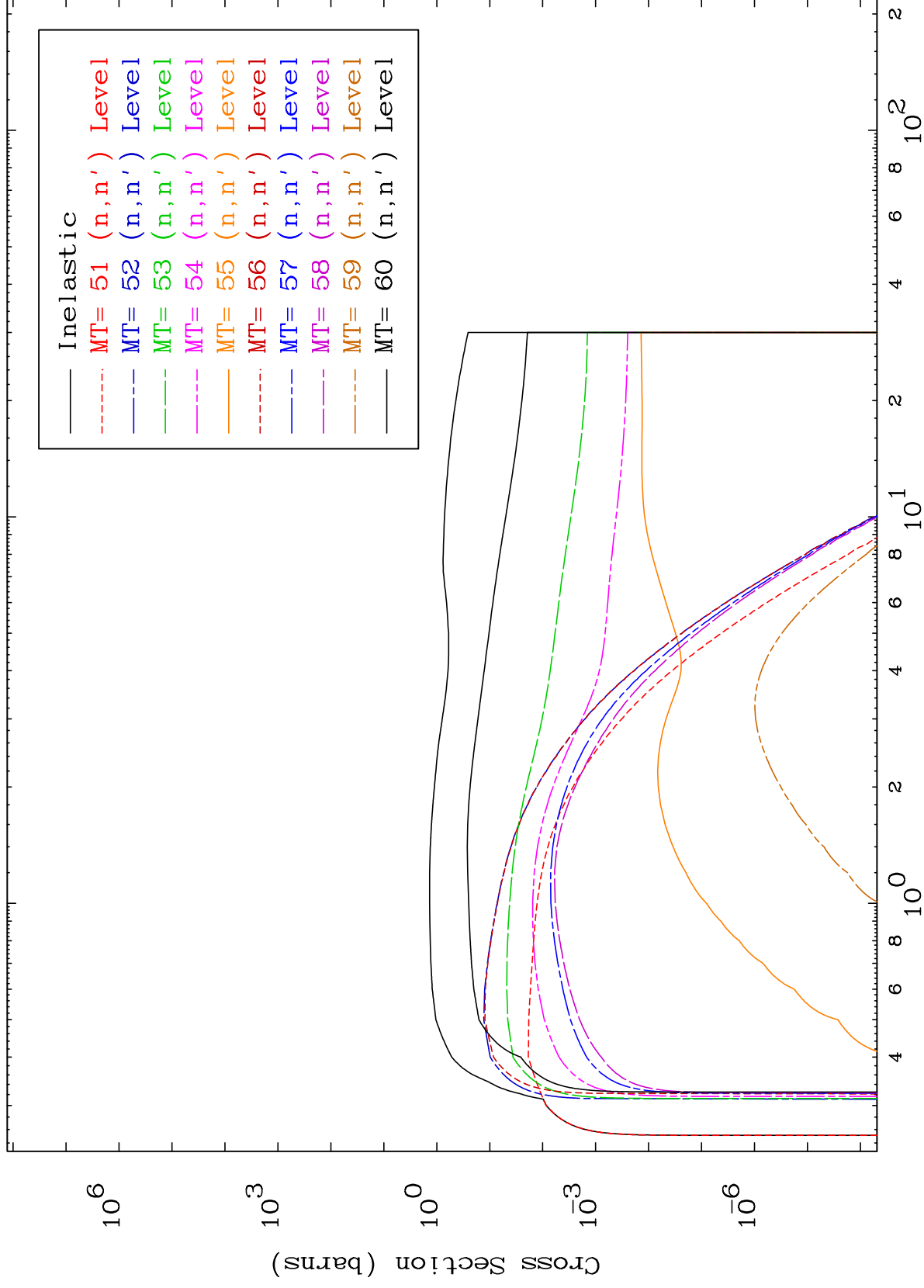


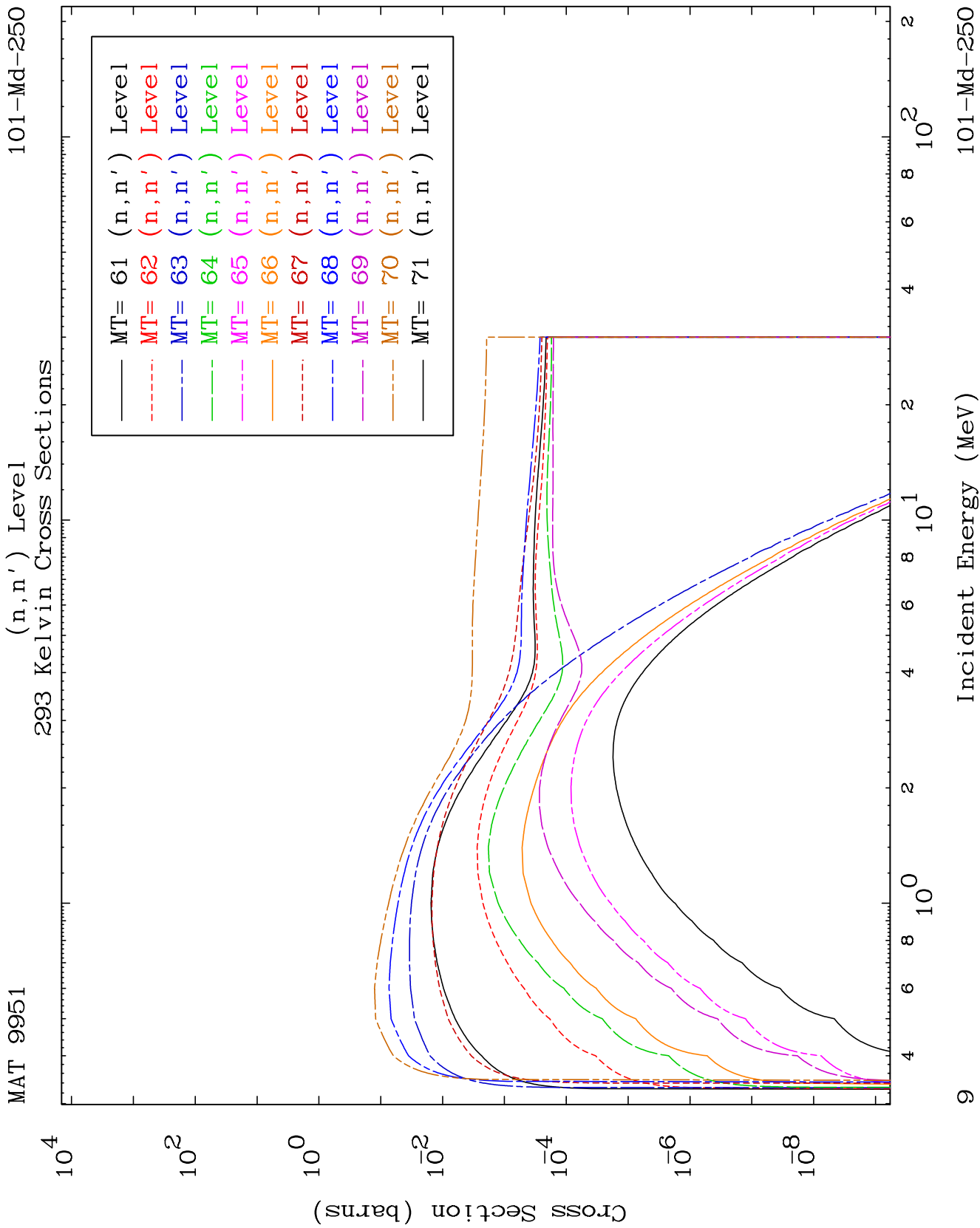
MAT 9951

(n,n') Level

101-Md-250

293 Kelvin Cross Sections



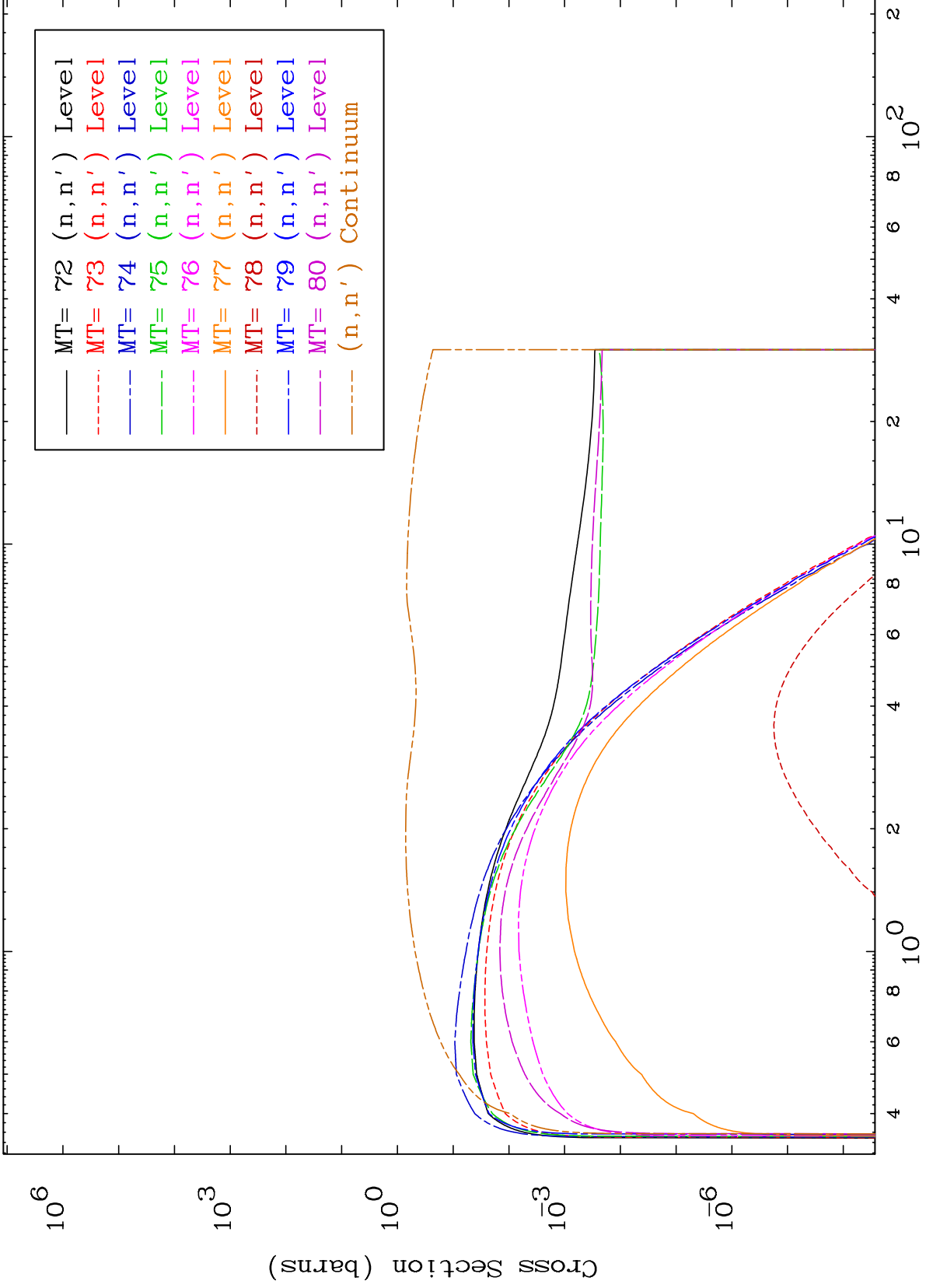


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(n,n') Level

101-Md-250

293 Kelvin Cross Sections



10

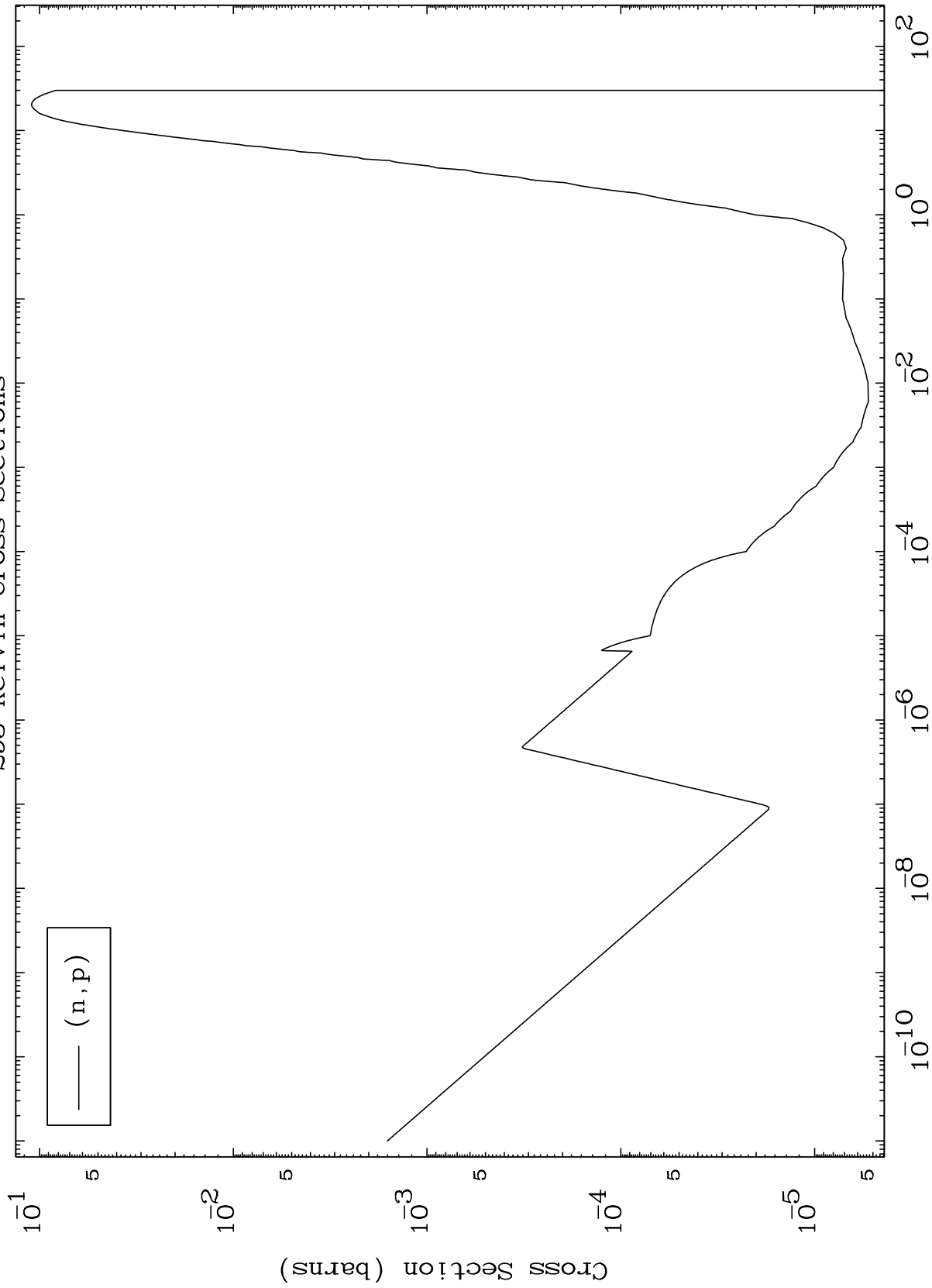
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,p) Levels  
293 Kelvin Cross Sections

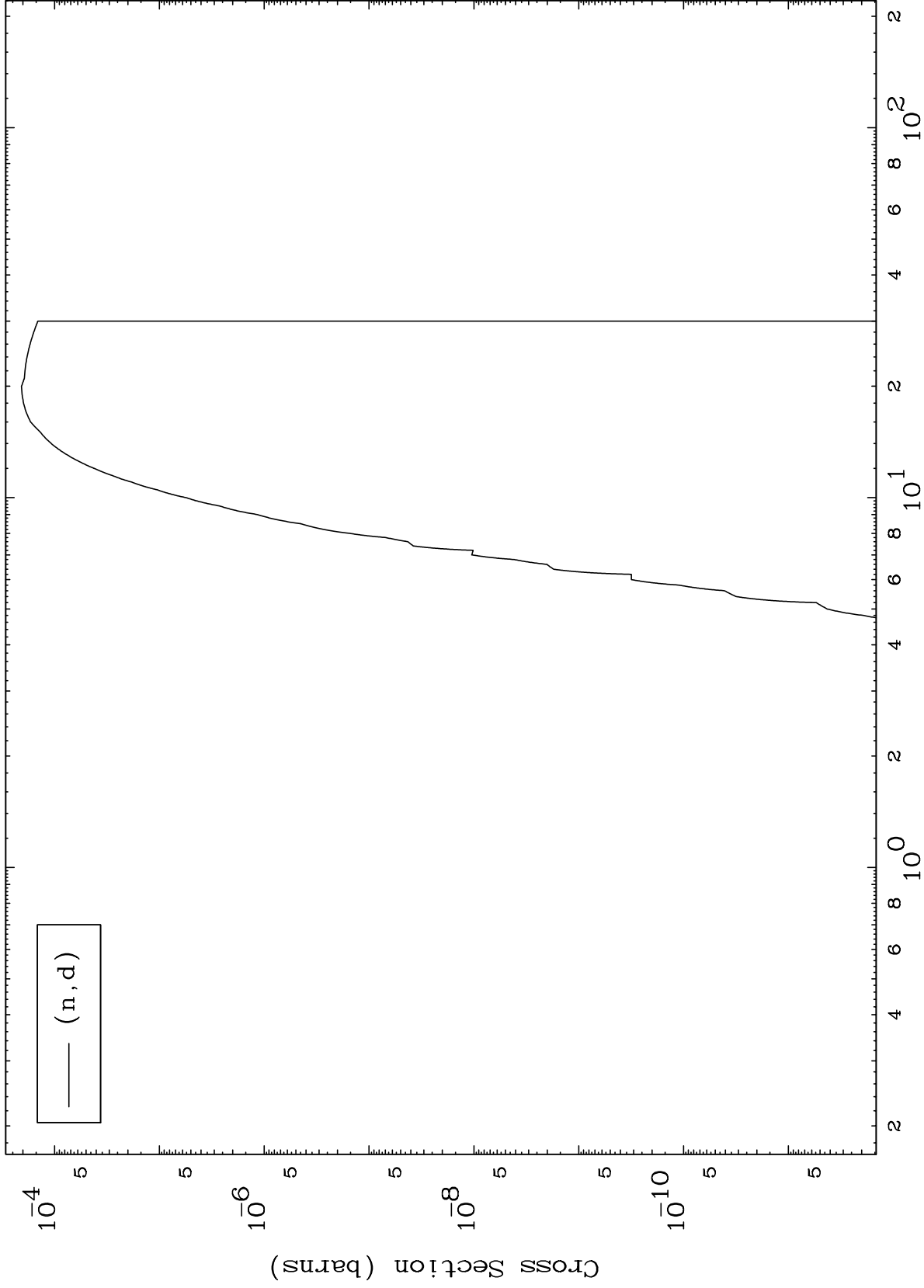
101-Md-250



MAT 9951

(n,d) Levels  
293 Kelvin Cross Sections

101-Md-250



12

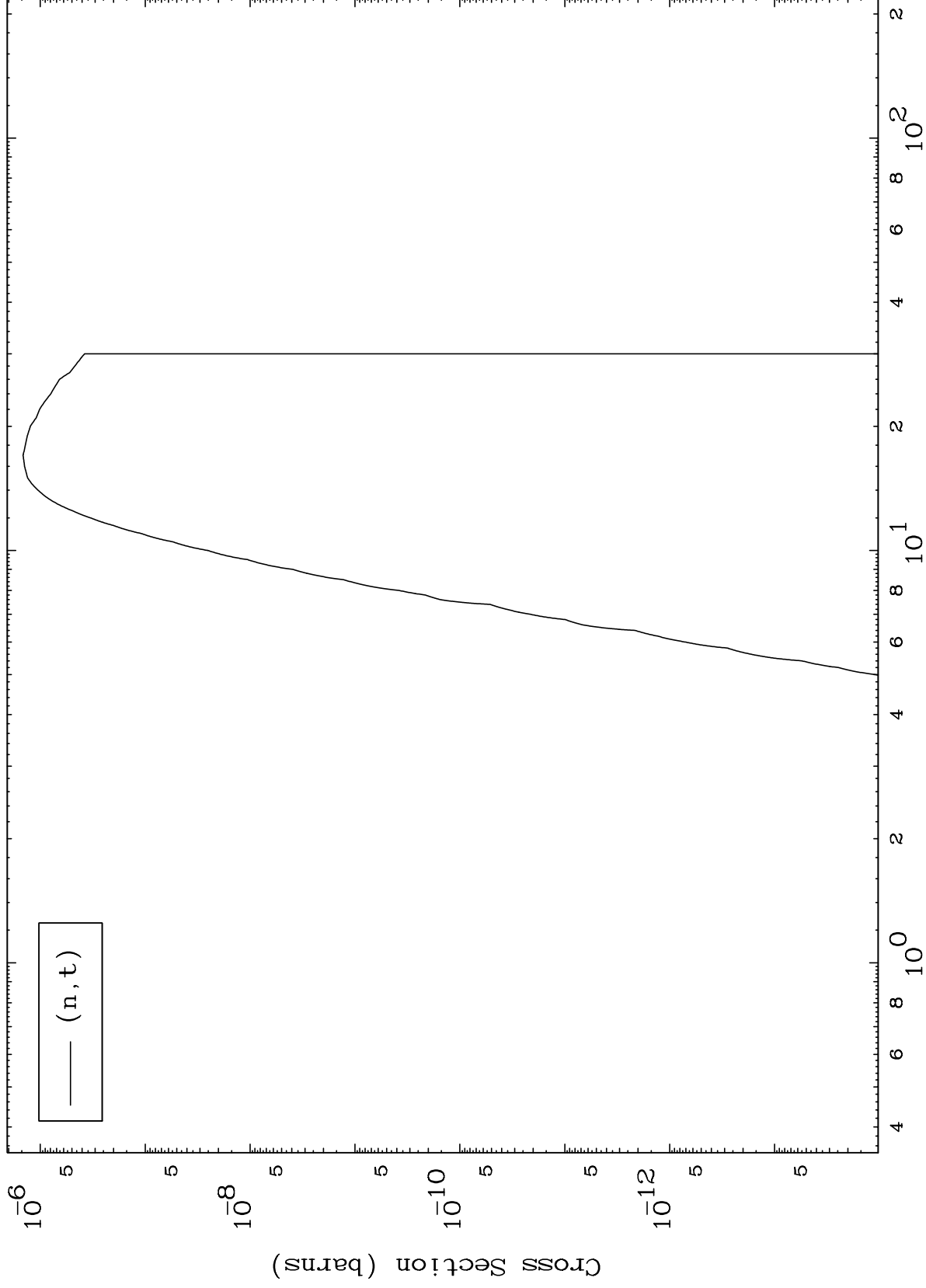
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,t) Levels  
293 Kelvin Cross Sections

101-Md-250



13

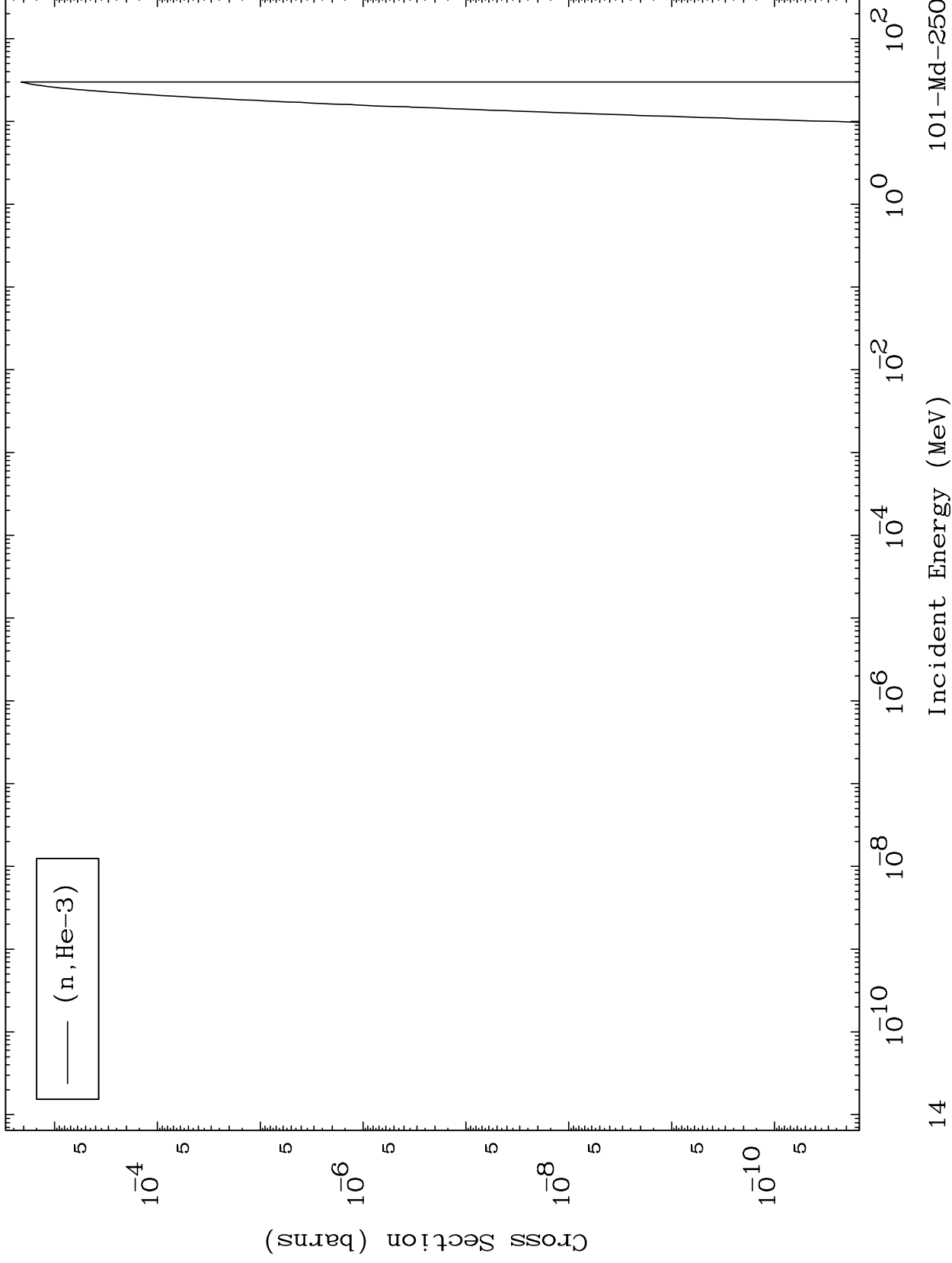
Incident Energy (MeV)

101-Md-250

MAT 9951

(n,He3) Levels  
293 Kelvin Cross Sections

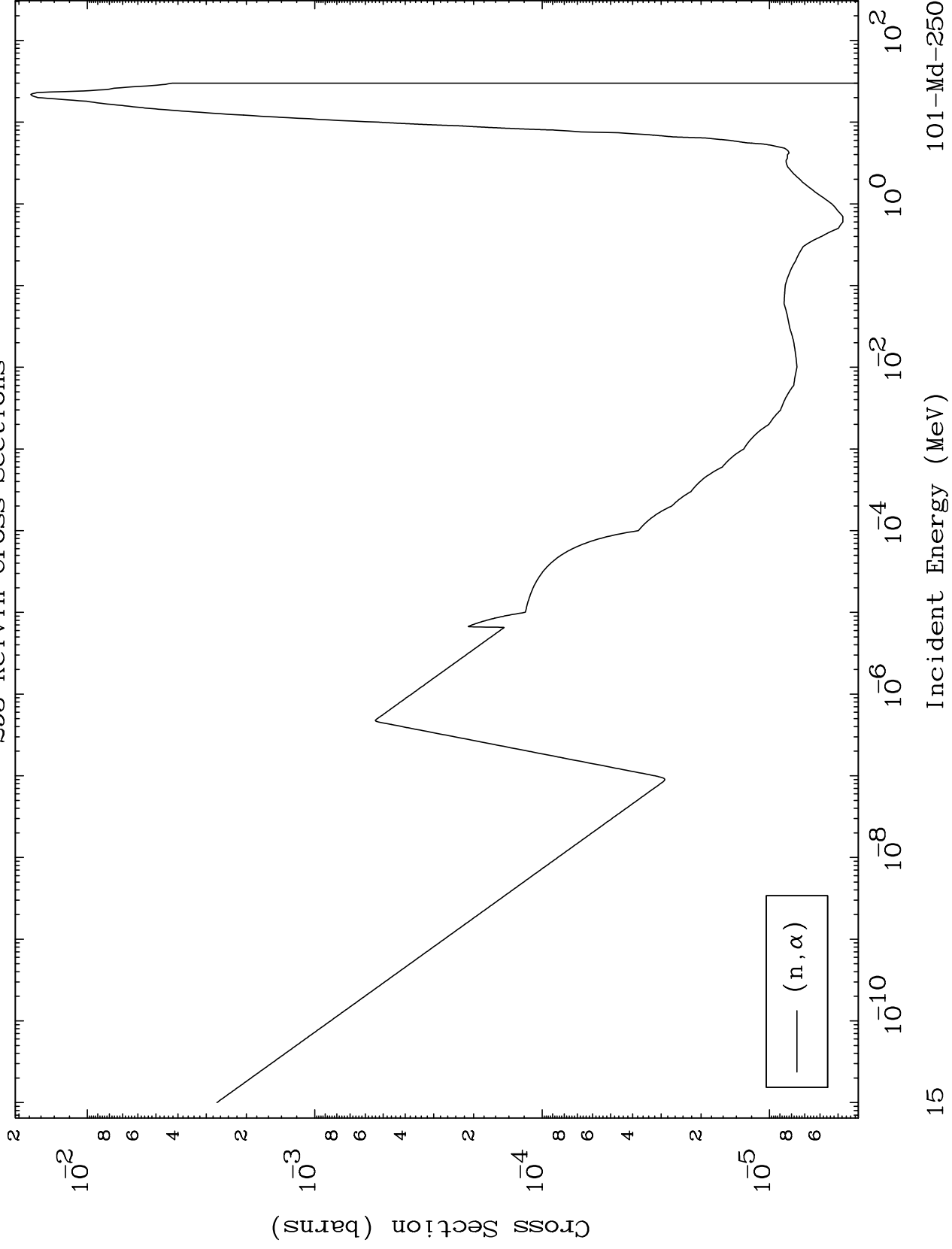
101-Md-250



MAT 9951

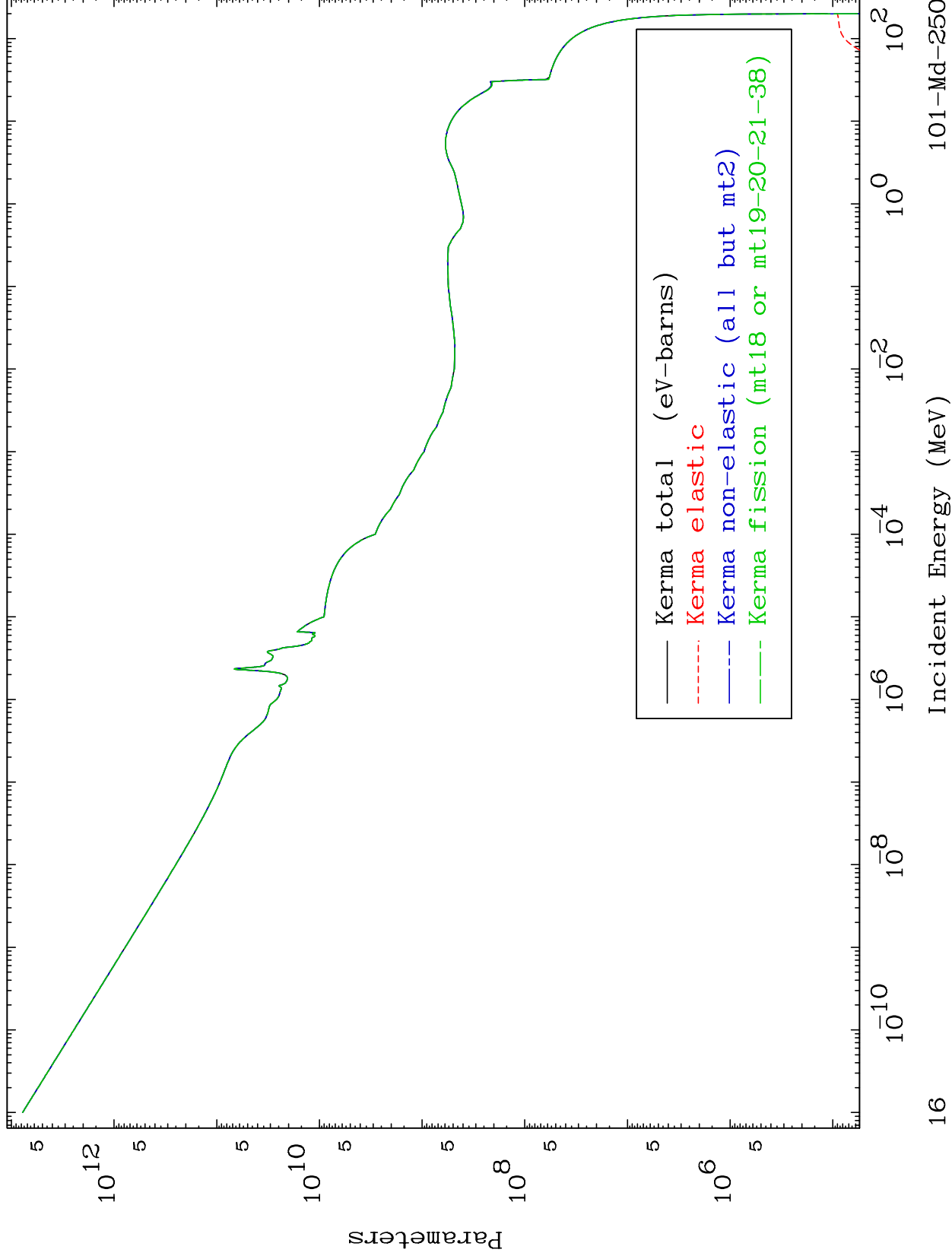
(n,α) Levels  
293 Kelvin Cross Sections

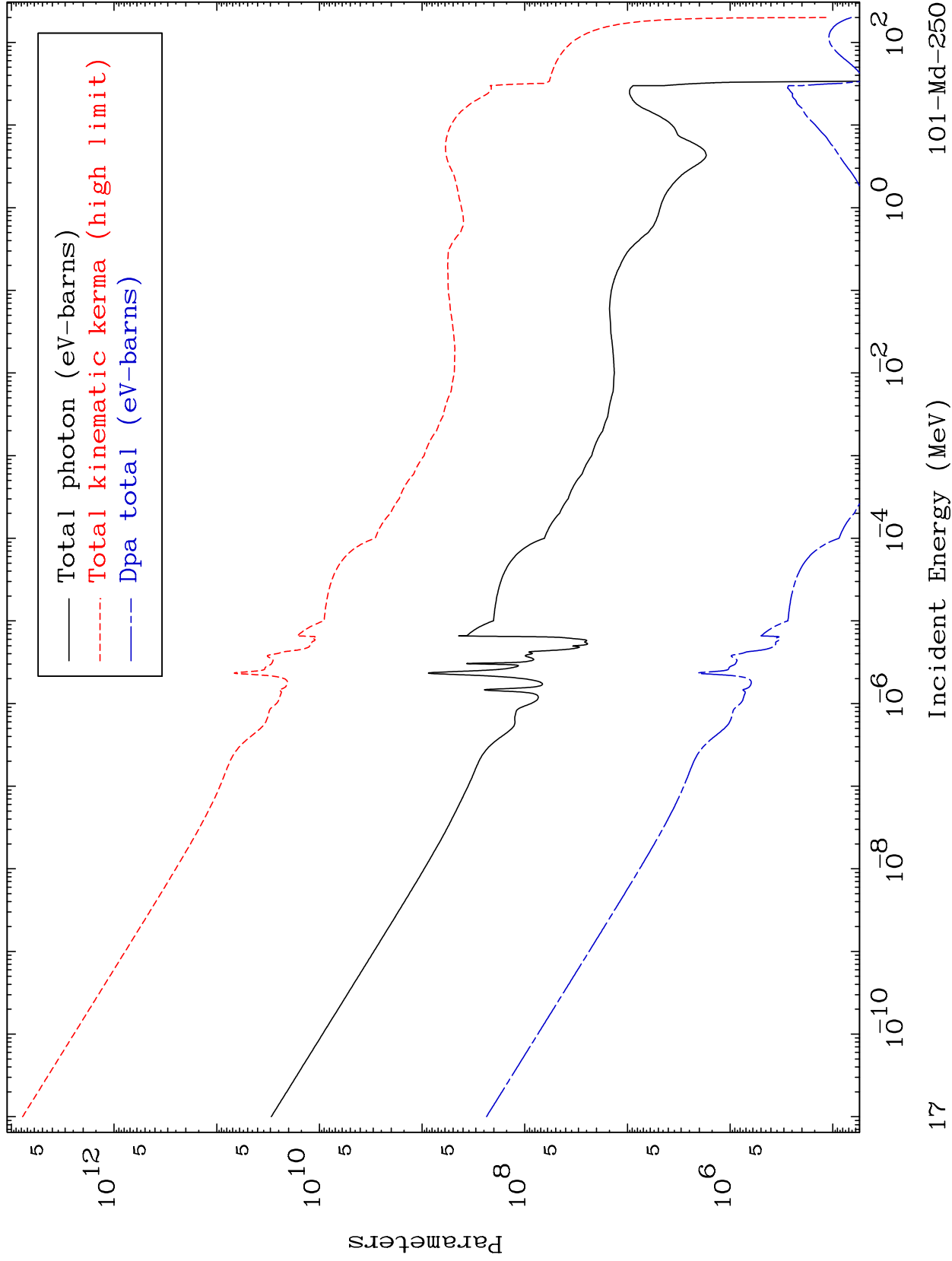
101-Md-250

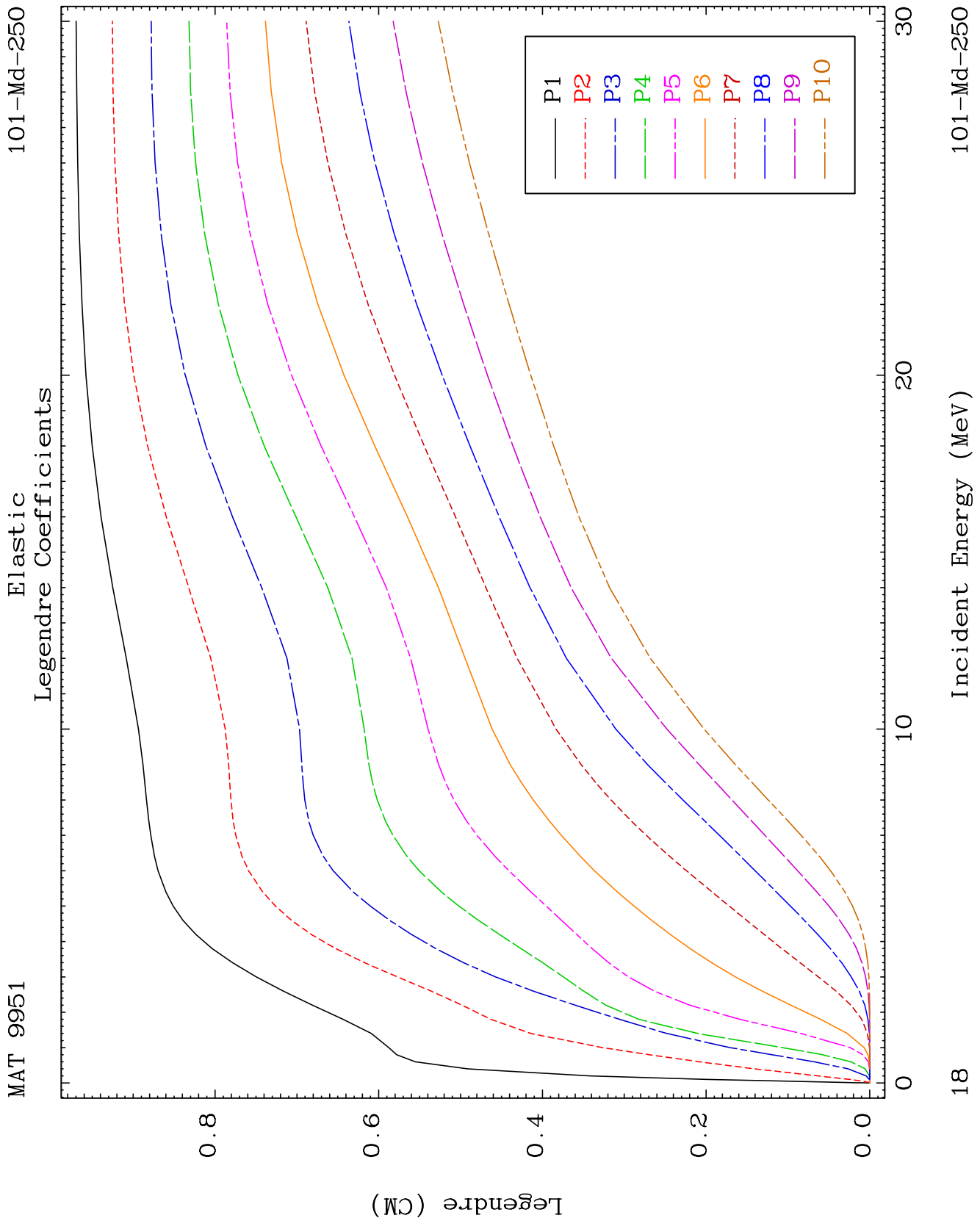


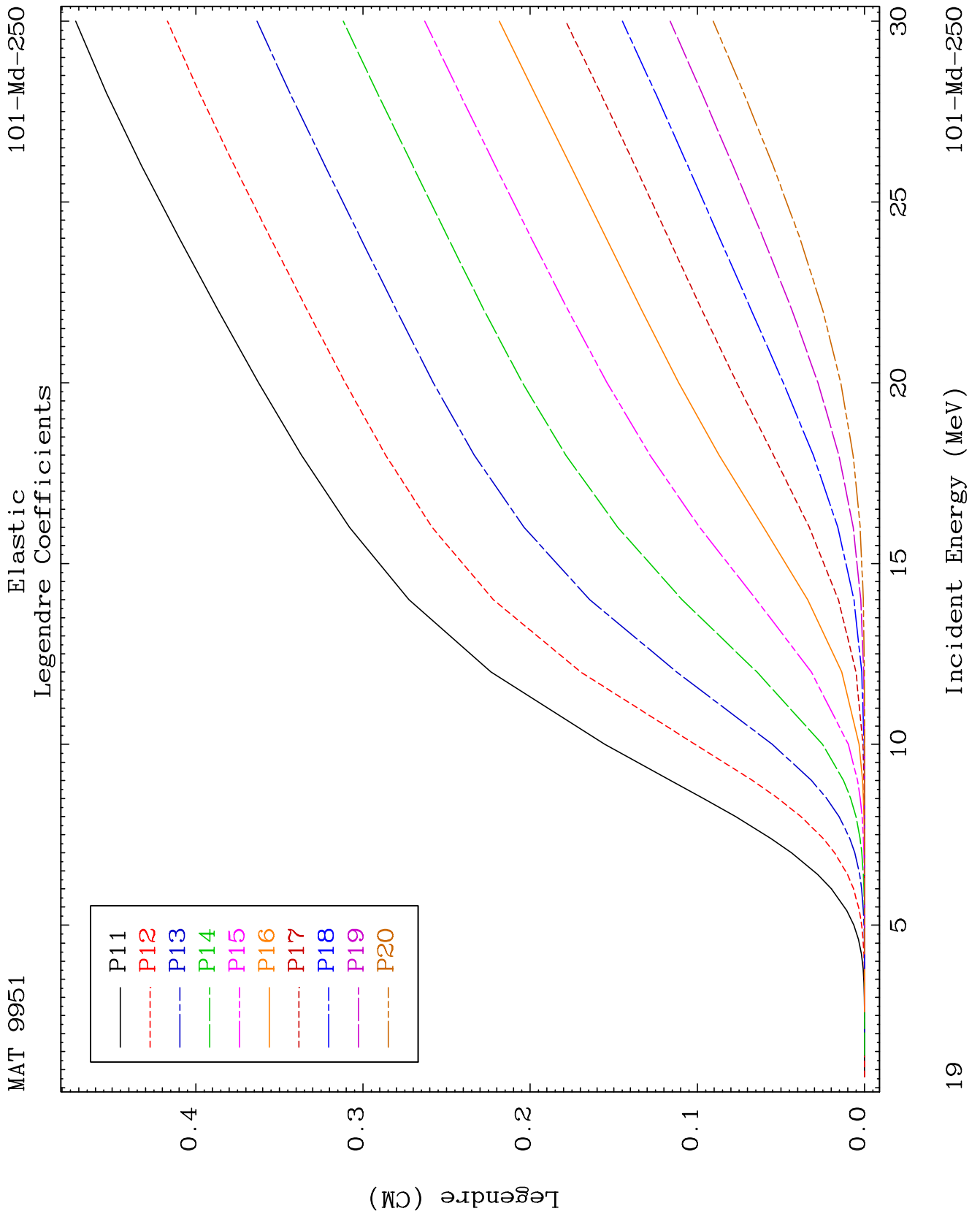
15







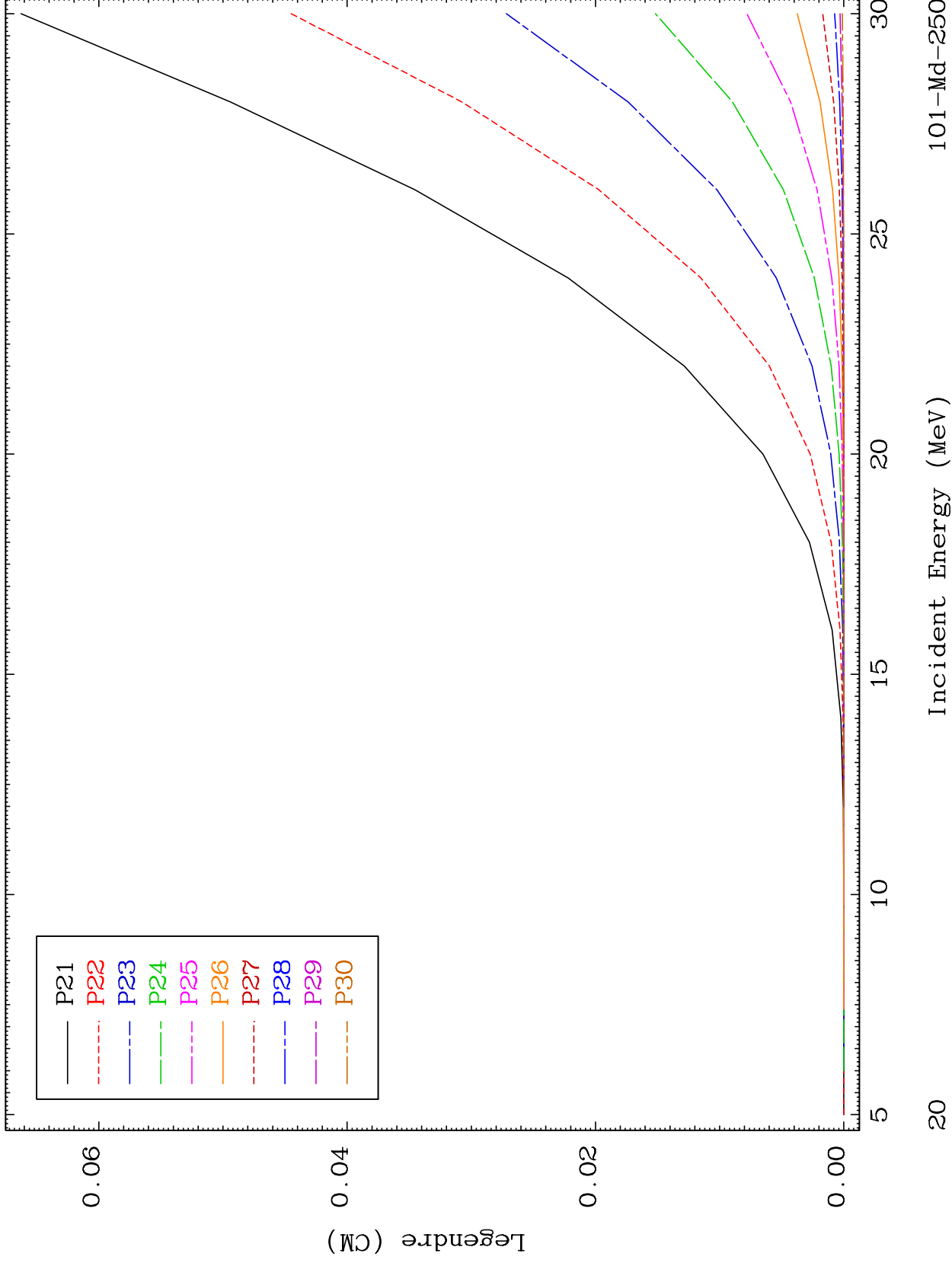


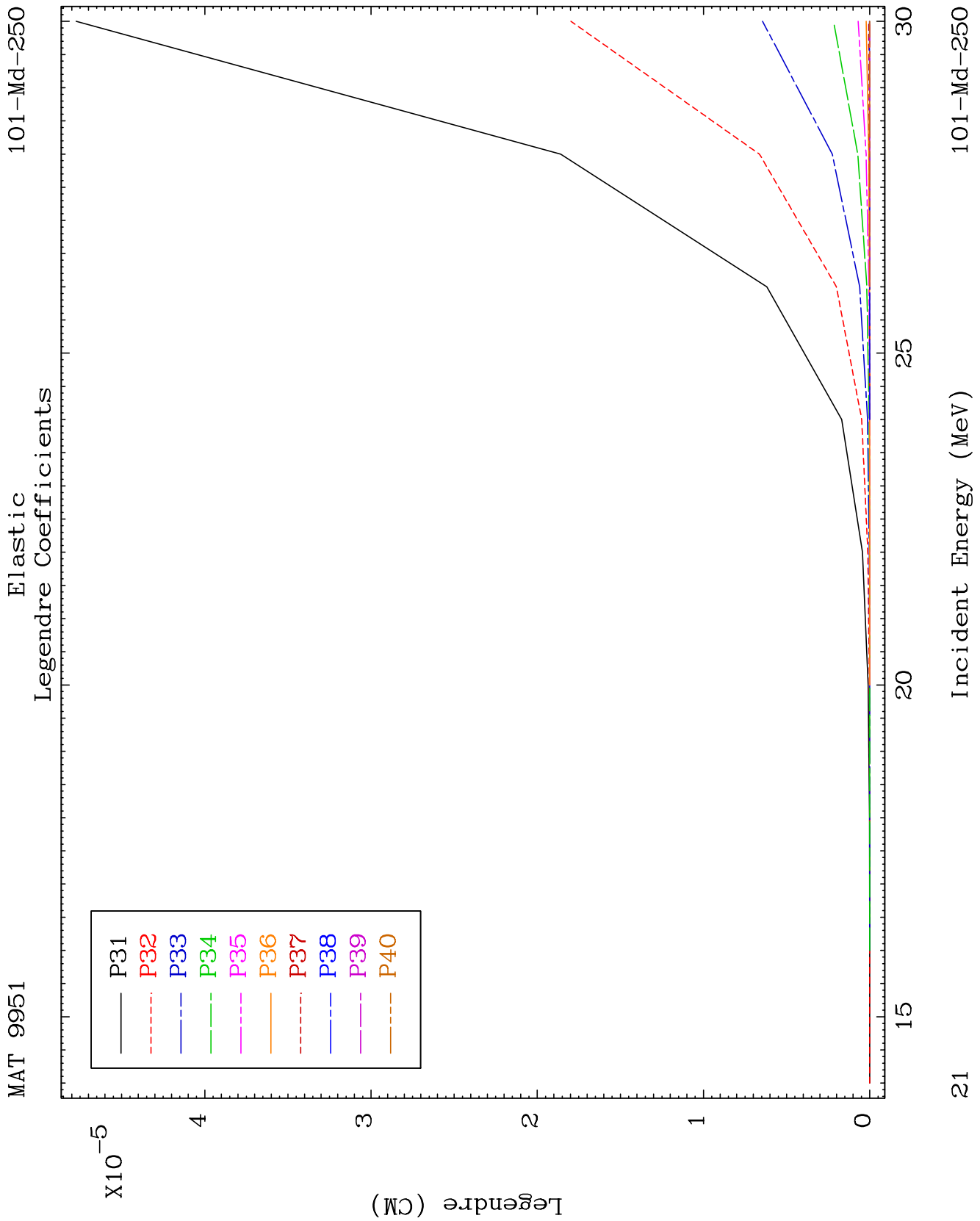


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Elastic  
Legendre Coefficients

101-Md-250

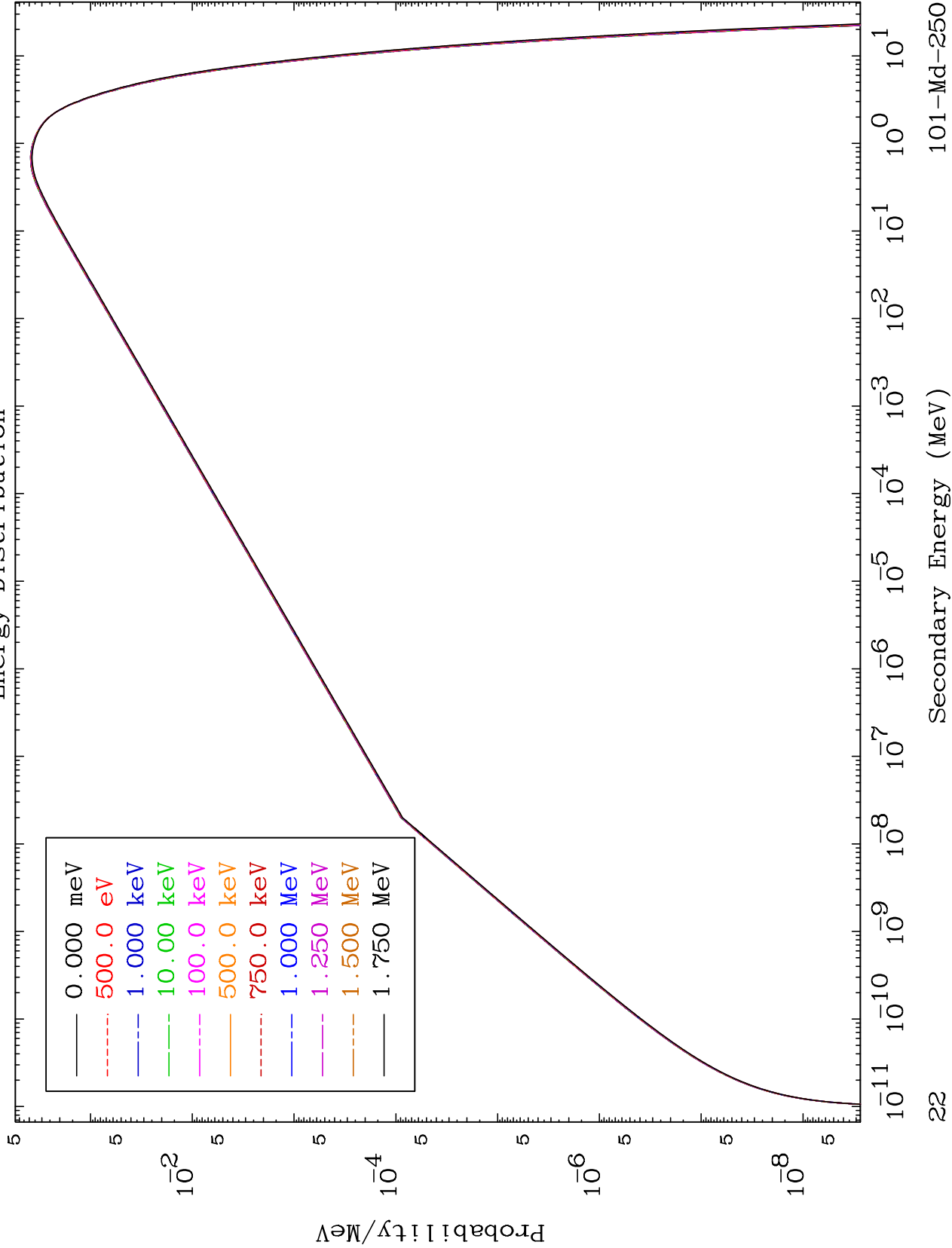




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### Fission Energy Distribution

101-Md-250



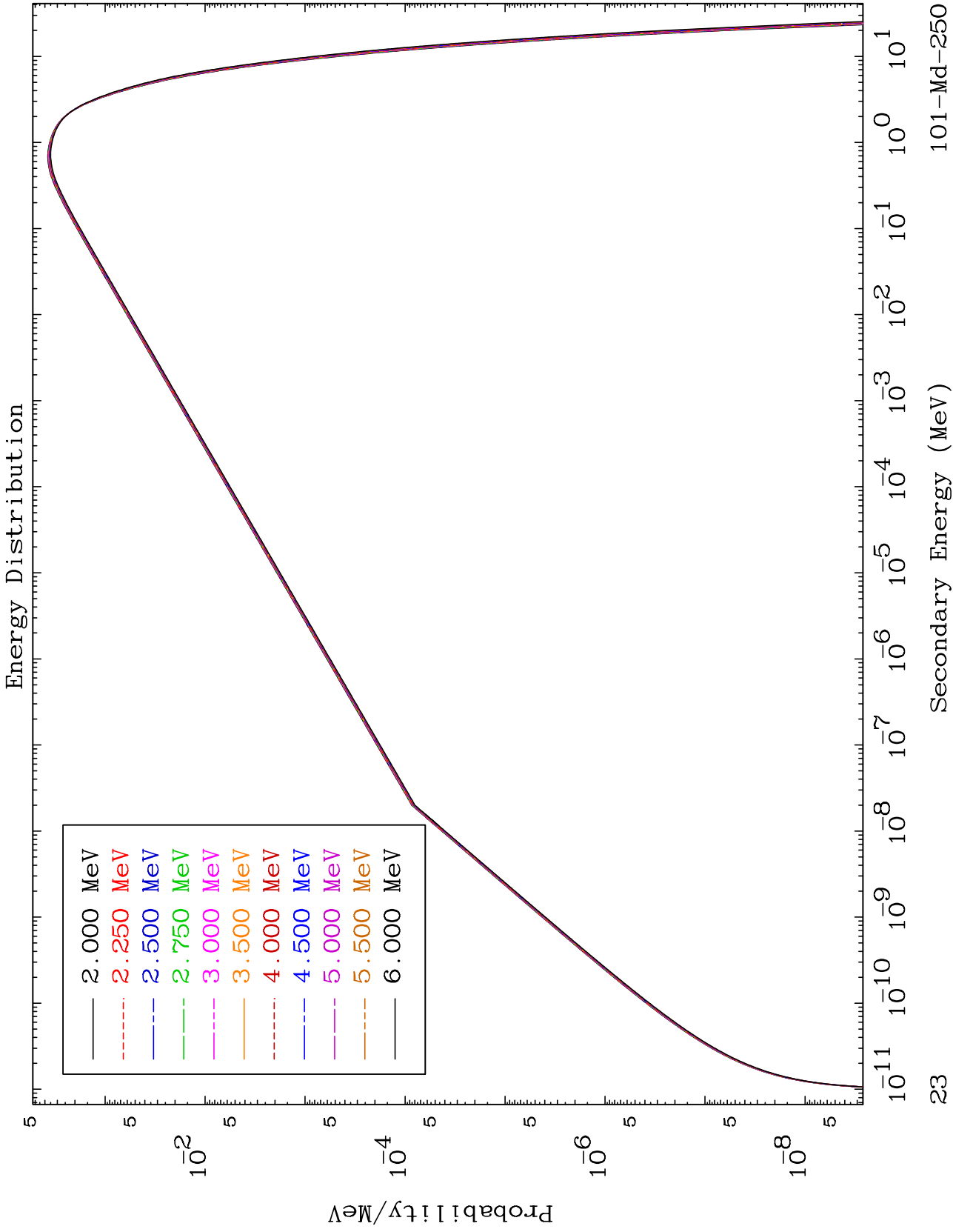
101-Md-250

Secondary Energy (MeV)

MAT 9951

Fission

101-Md-250

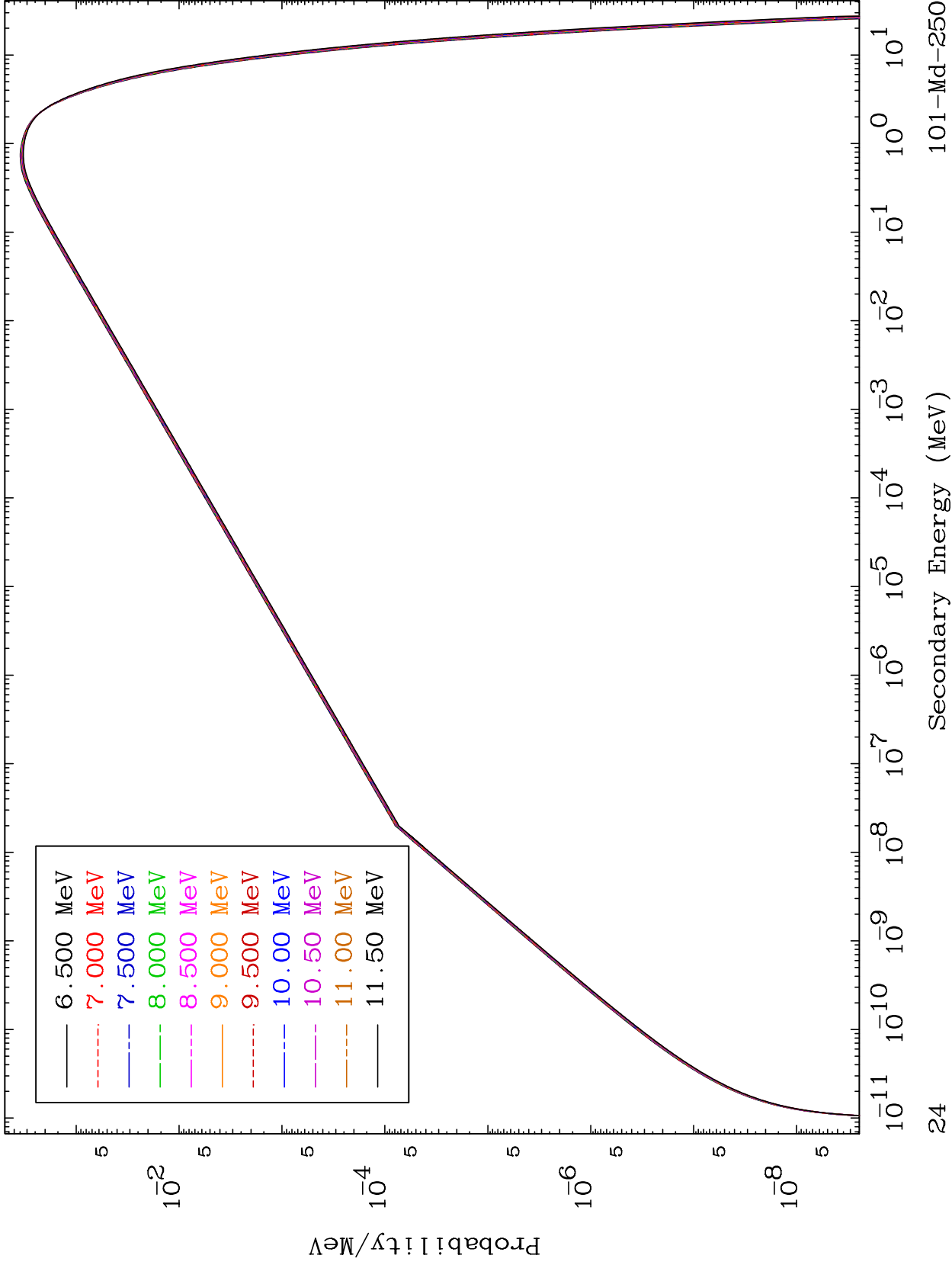




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Fission  
Energy Distribution

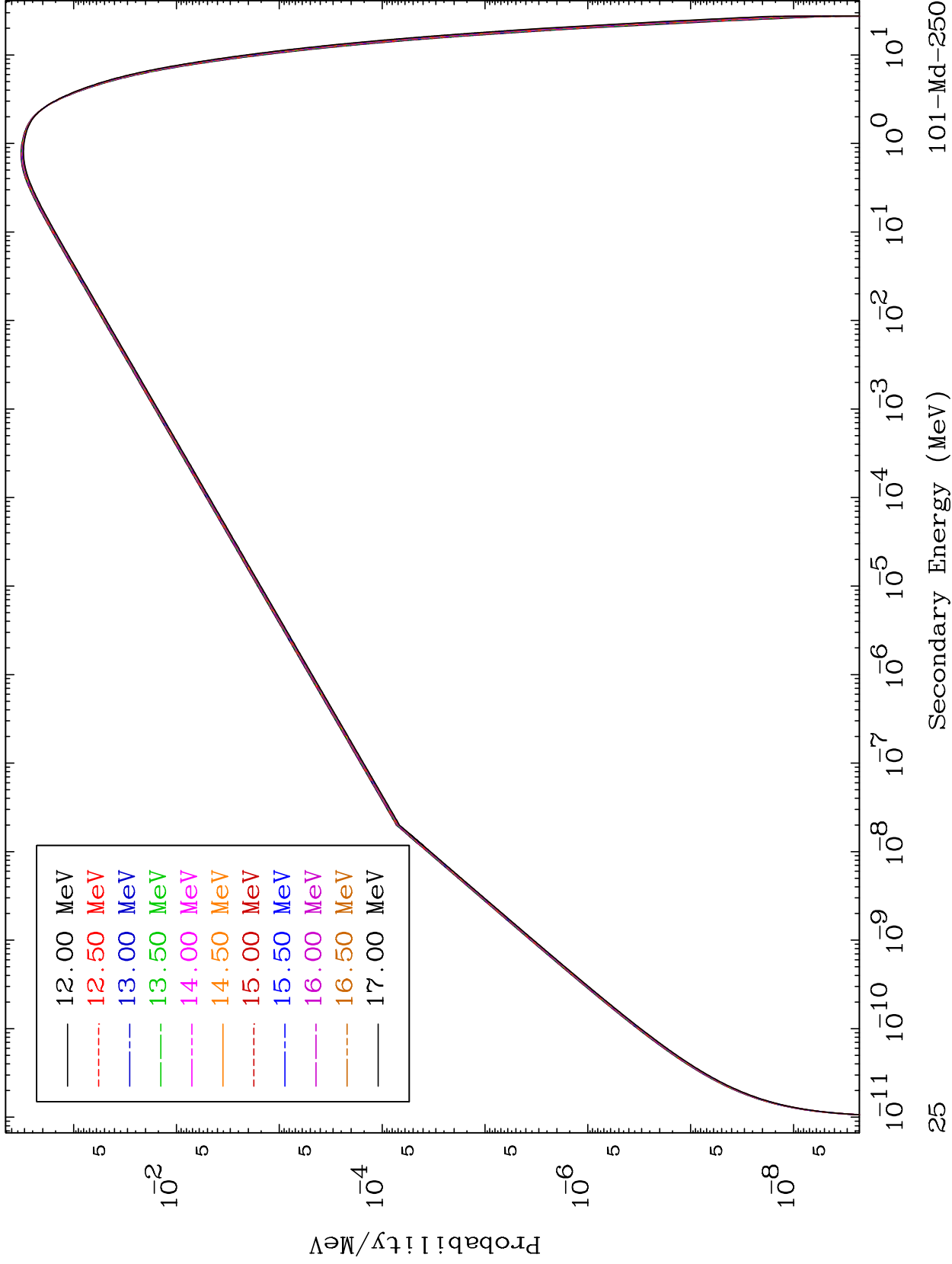
101-Md-250



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### Fission Energy Distribution

101-Md-250



25

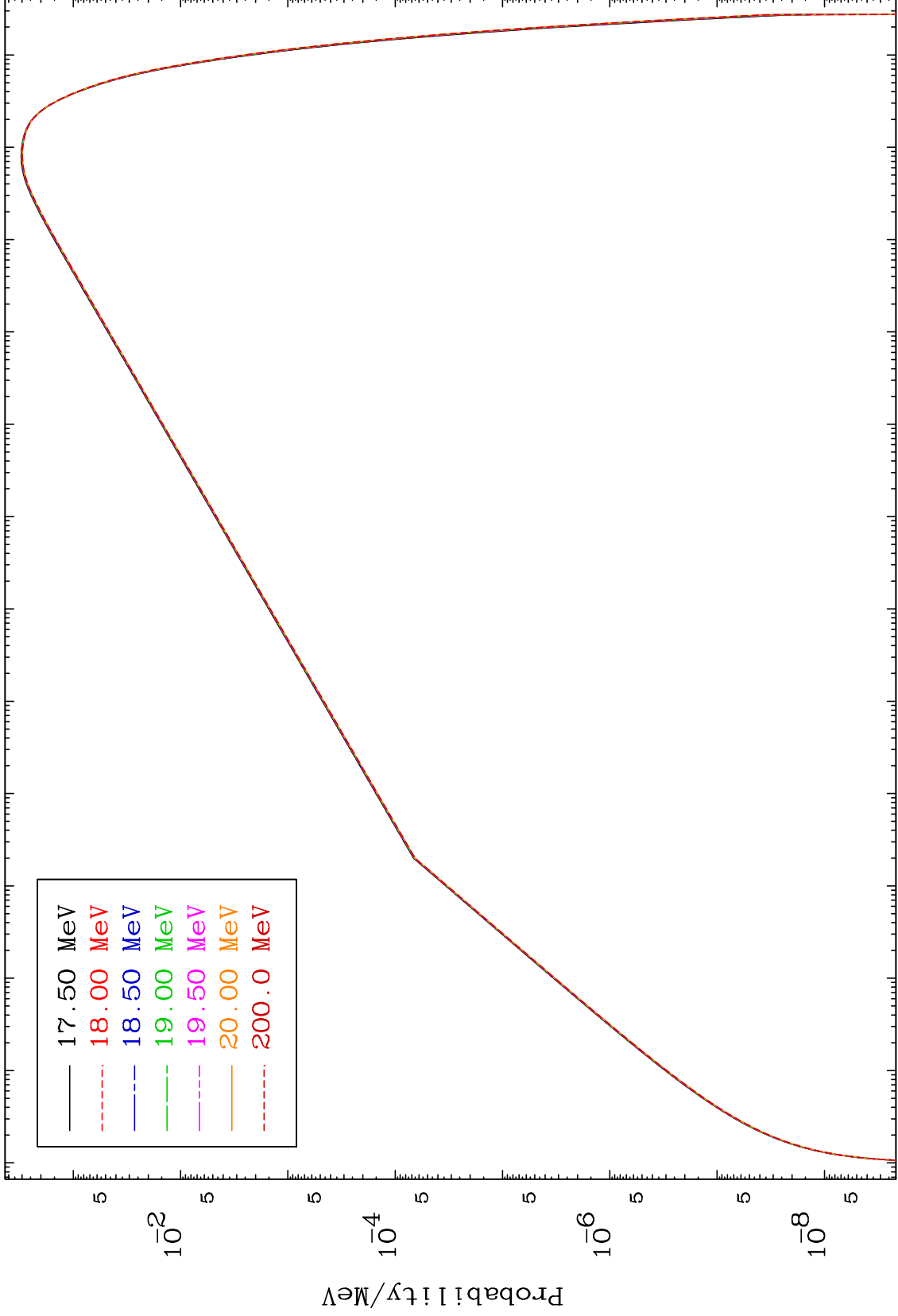
101-Md-250

MAT 9951

Fission

101-Md-250

Energy Distribution

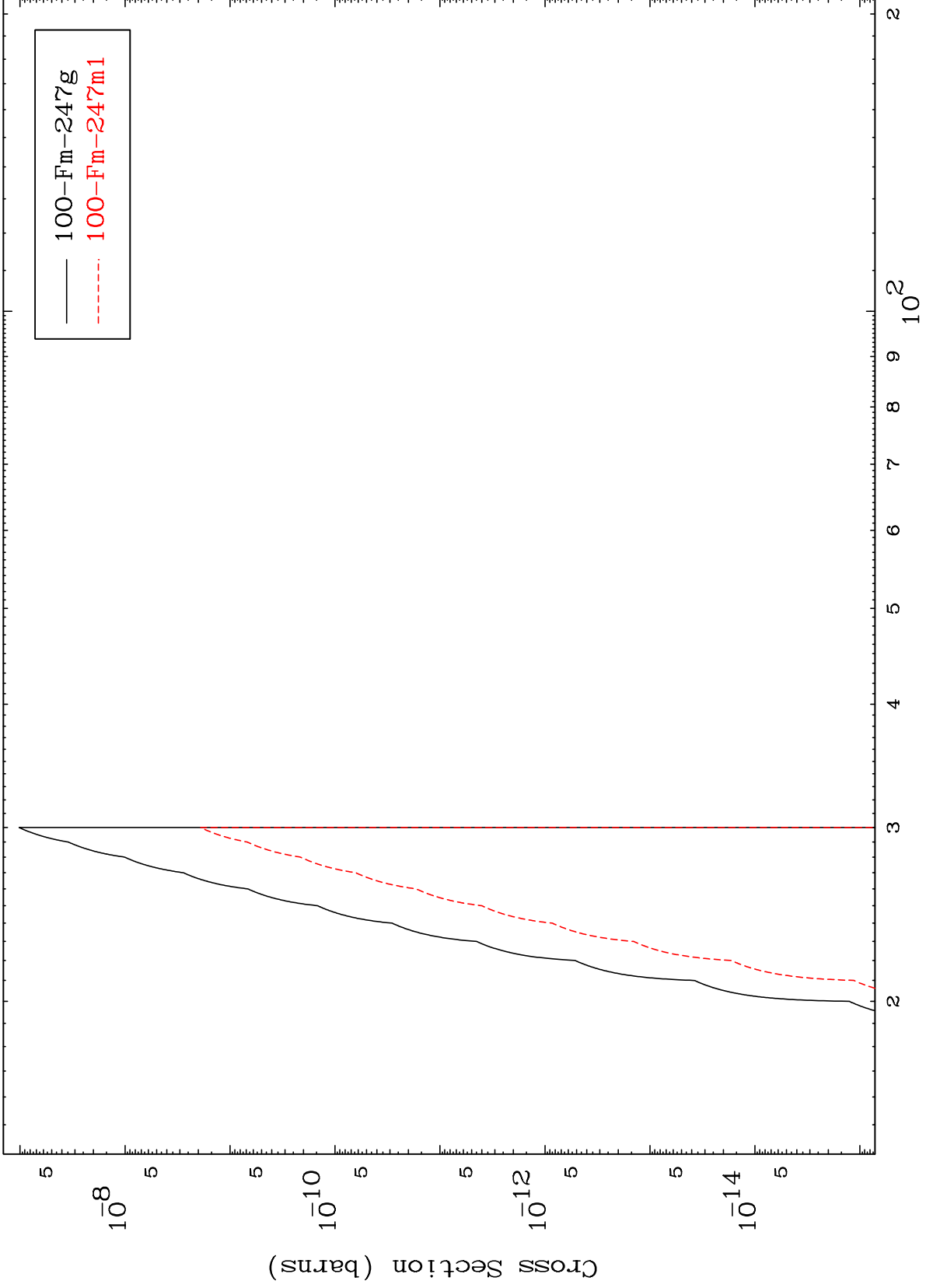


10<sup>-11</sup> 10<sup>-10</sup> 10<sup>-9</sup> 10<sup>-8</sup> 10<sup>-7</sup> 10<sup>-6</sup> 10<sup>-5</sup> 10<sup>-4</sup> 10<sup>-3</sup> 10<sup>-2</sup> 10<sup>-1</sup> 10<sup>0</sup> 10<sup>1</sup>

101-Md-250

26

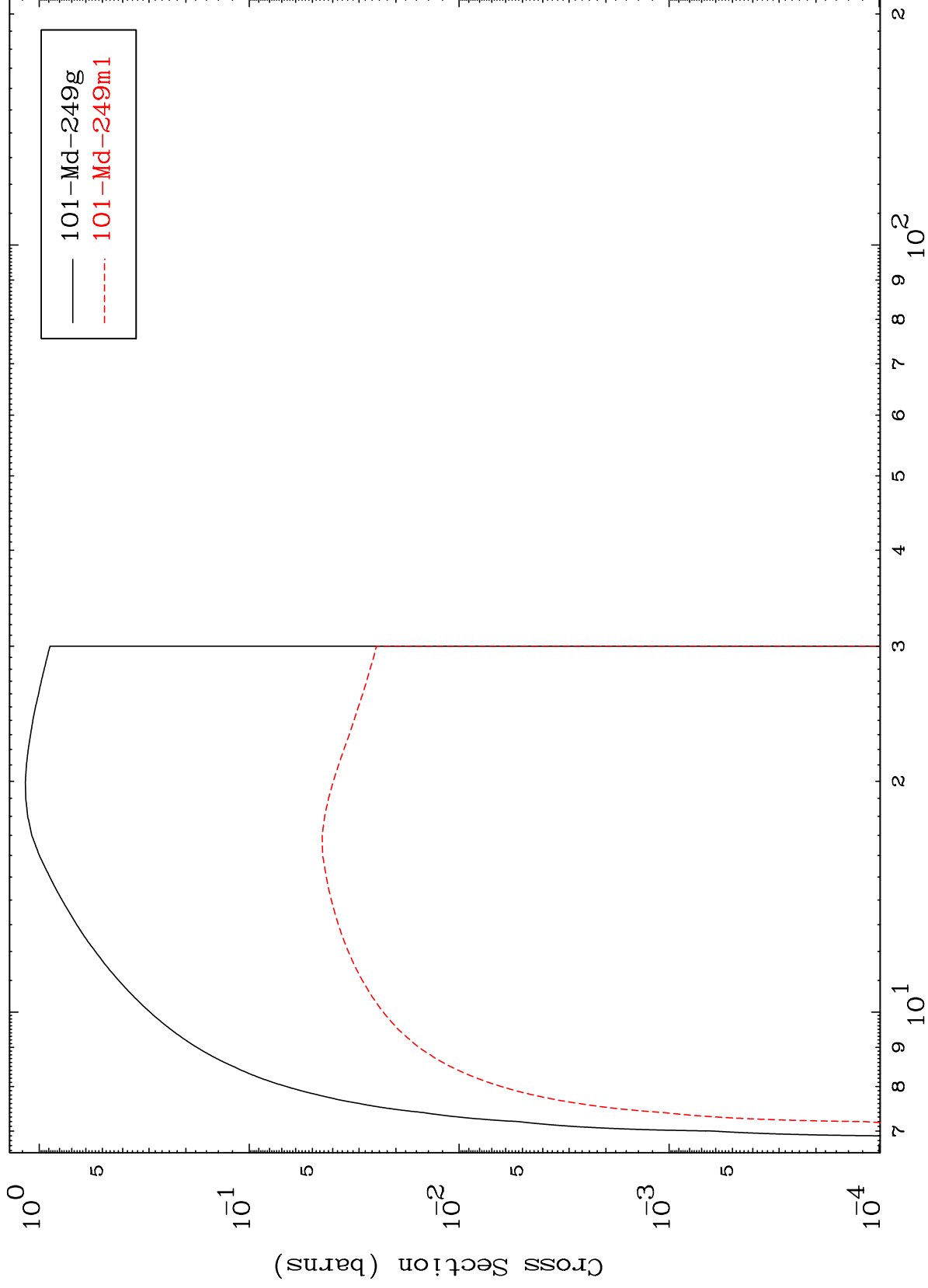
Radionuclide Production Cross Section



MAT 9951

101-Md-250

(n,2n)  
Radionuclide Production Cross Section



28

Incident Energy (MeV)

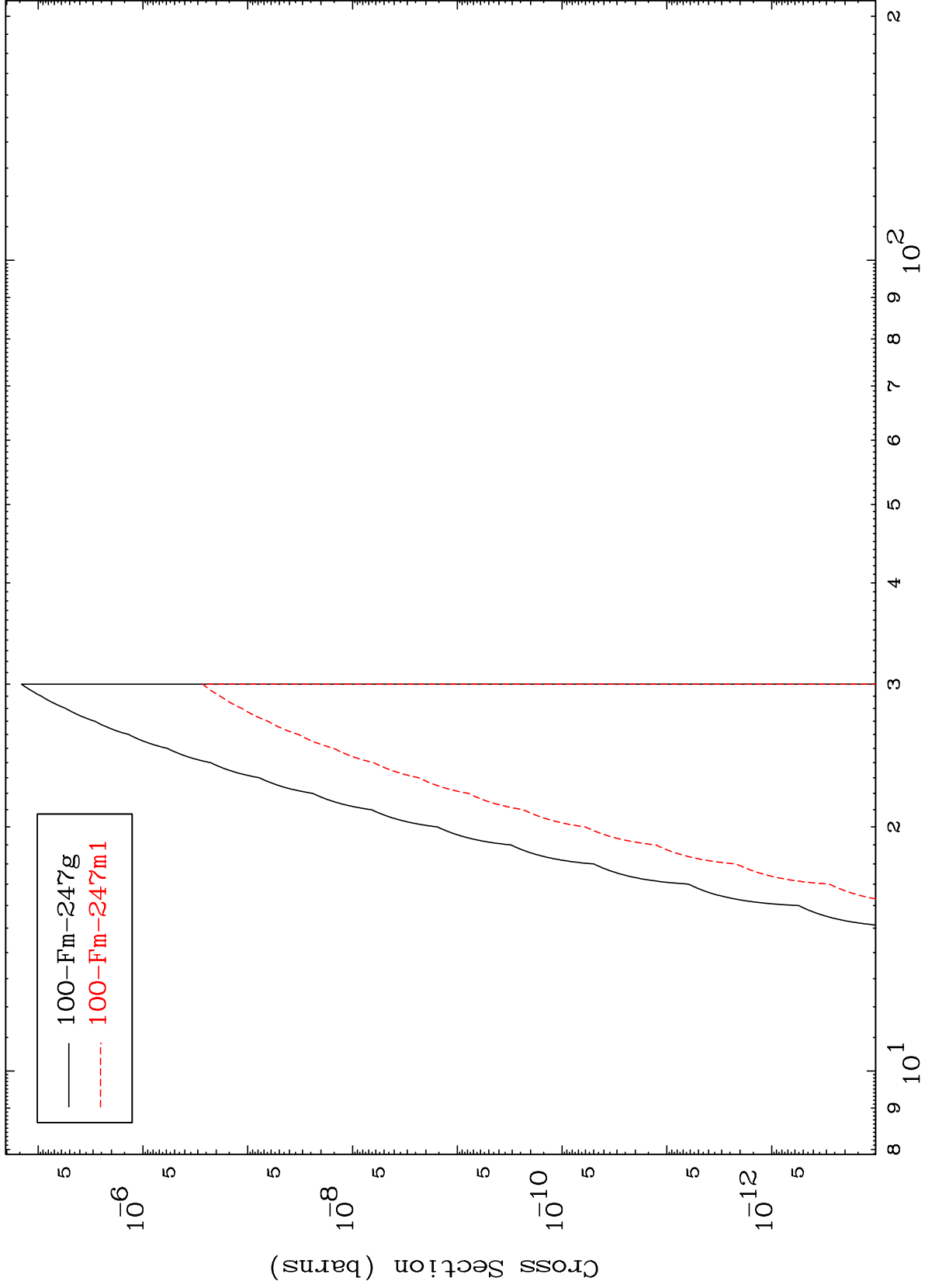
101-Md-250

MAT 9951

(n,n') t

101-Md-250

Radionuclide Production Cross Section



29

Incident Energy (MeV)

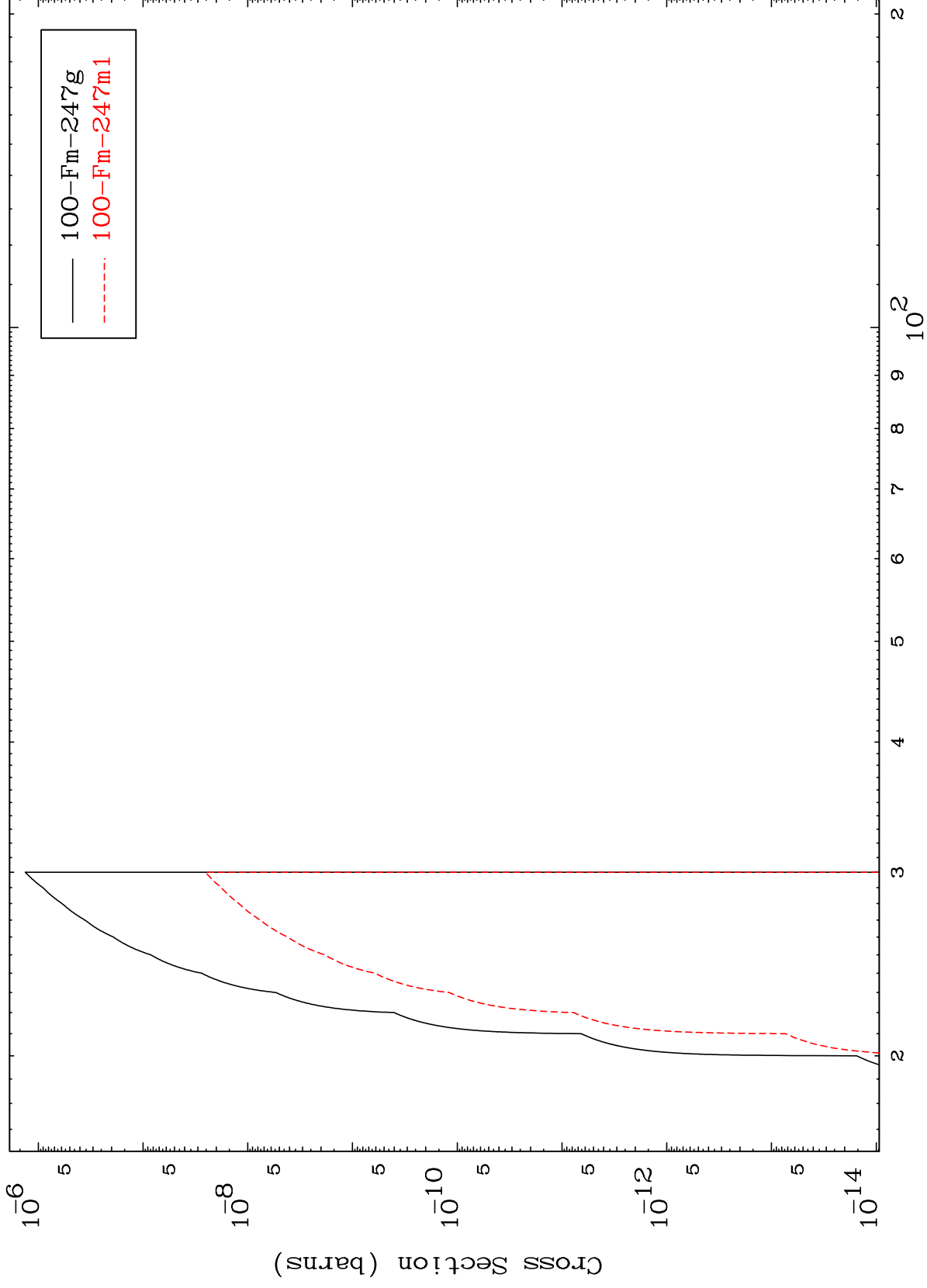
101-Md-250

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(n,3n) p

101-Md-250

Radionuclide Production Cross Section



30

Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

(n,p)  
Radionuclide Production Cross Section

