

Program EVALPLOT
(Version 2018-1)

by

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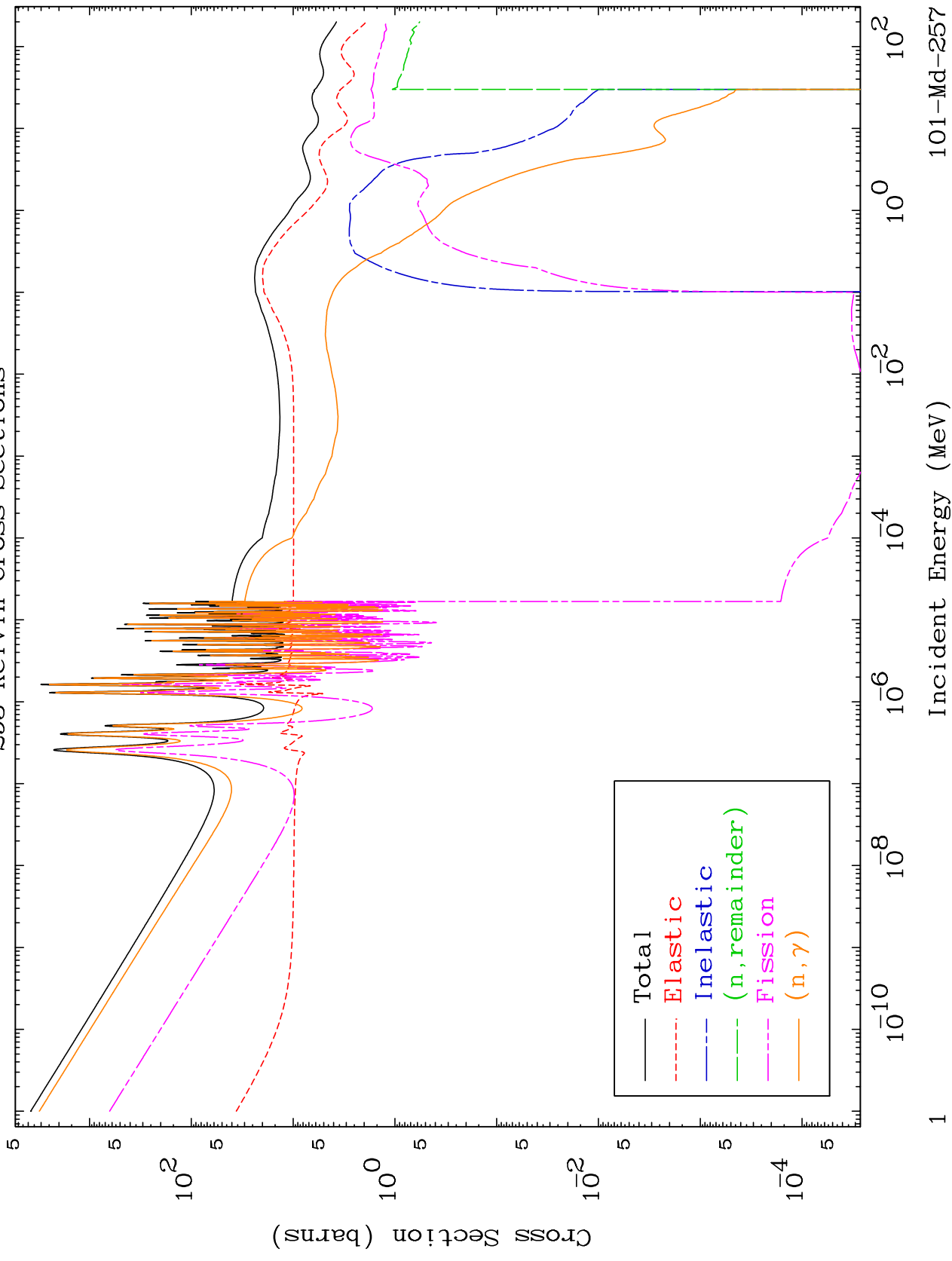
Press Mouse Button to Start

MAT 9958

Major

293 Kelvin Cross Sections

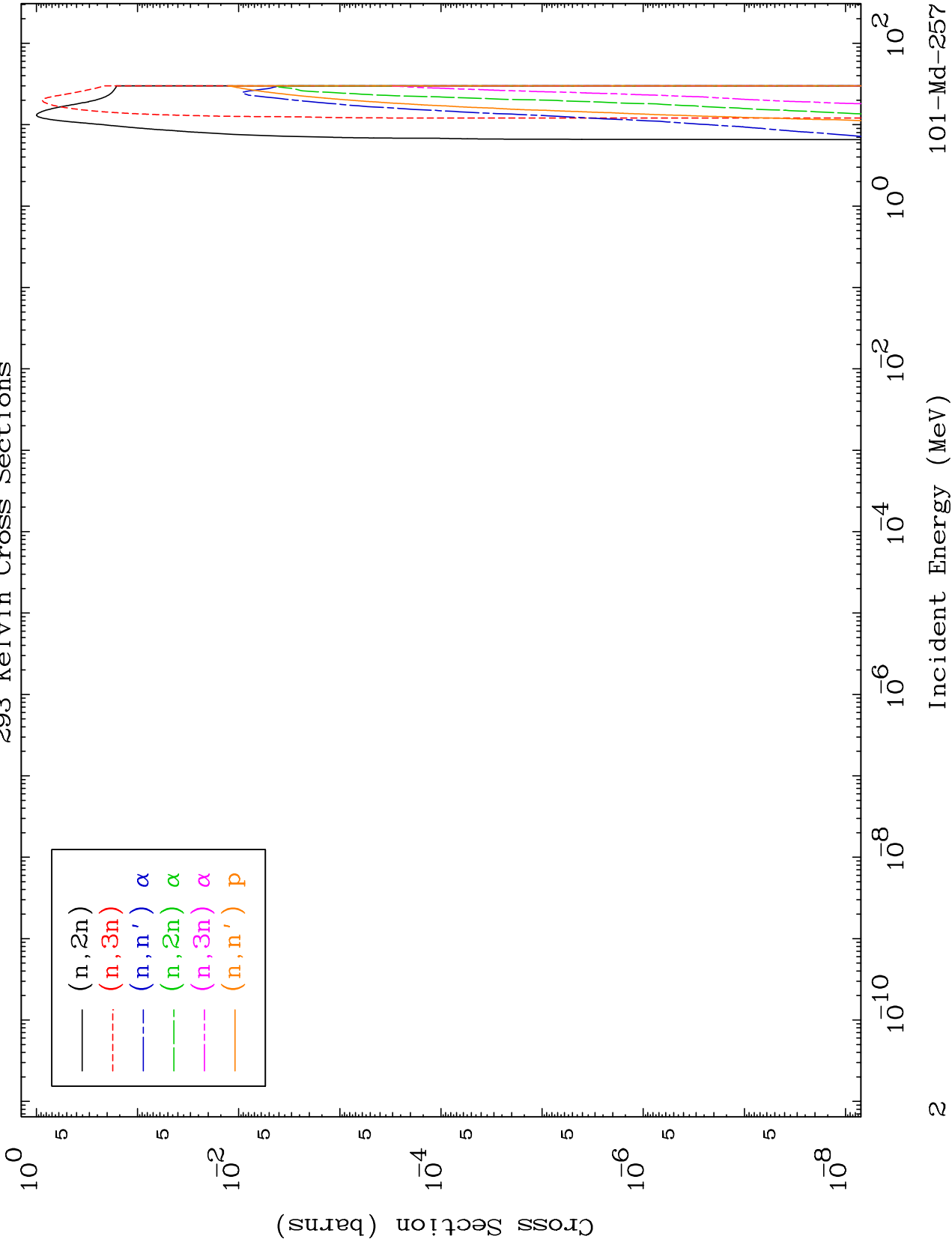
101-Md-257

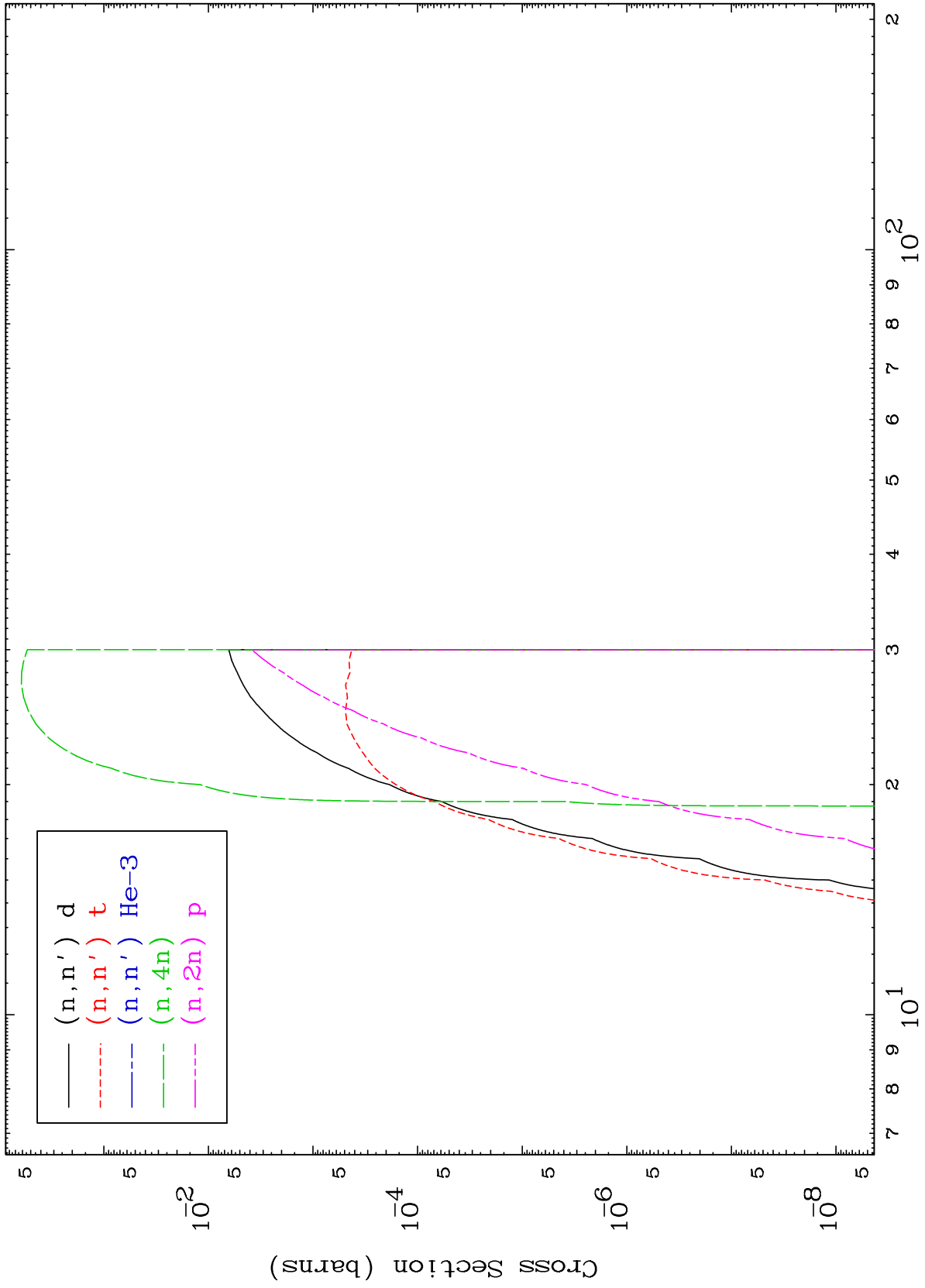


MAT 9958

Neutron Production
293 Kelvin Cross Sections

101-Md-257

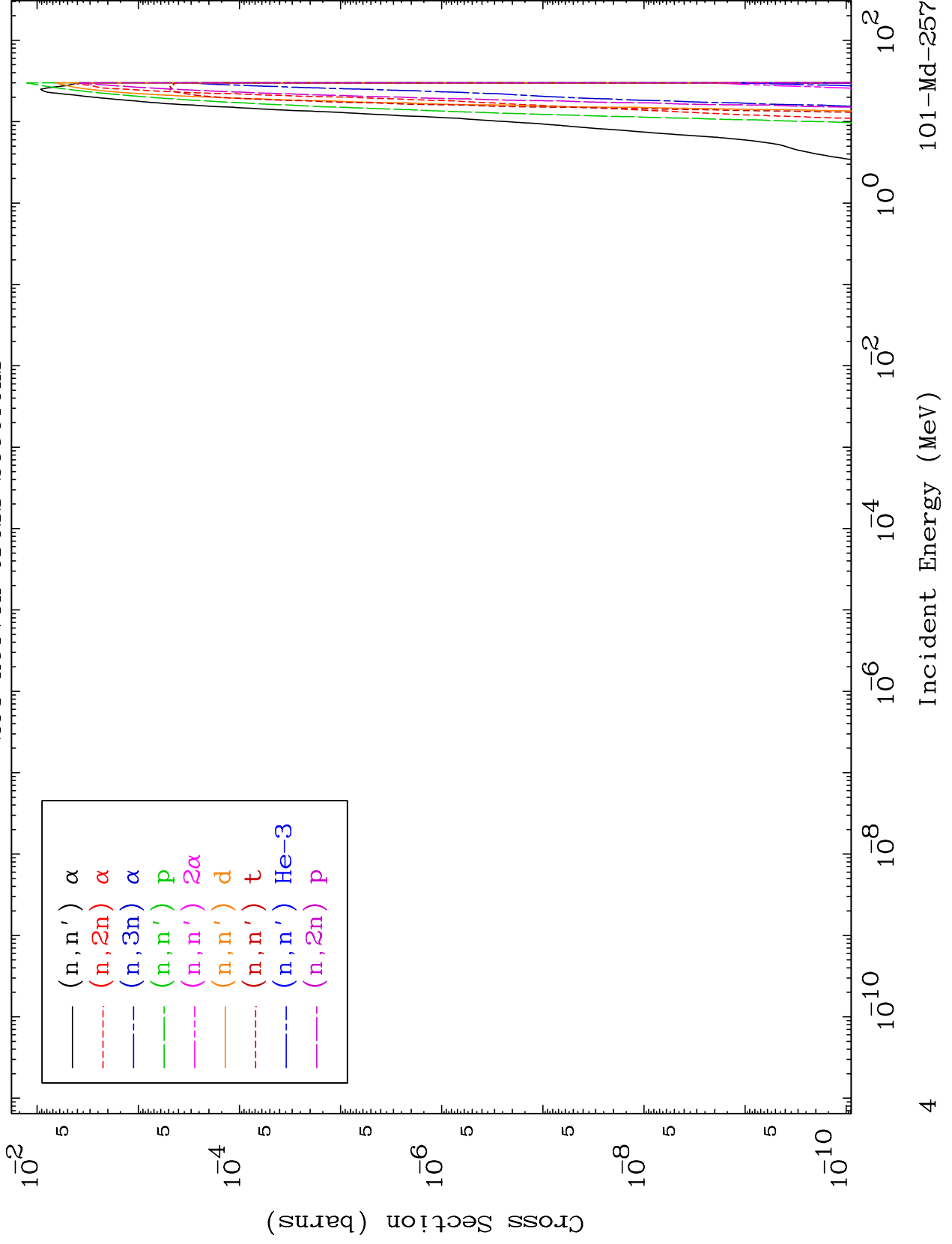




MAT 9958

Charged Particle
293 Kelvin Cross Sections

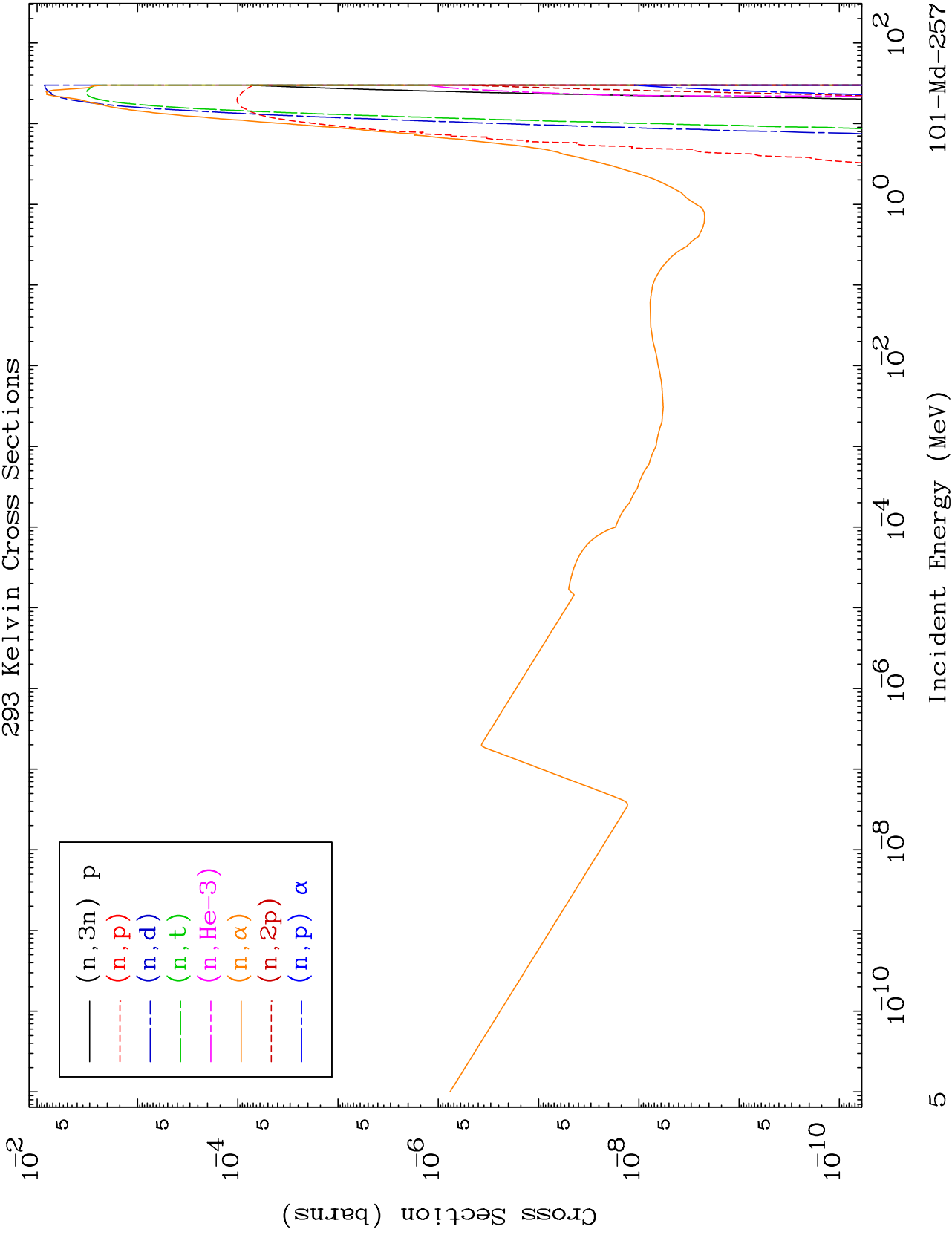
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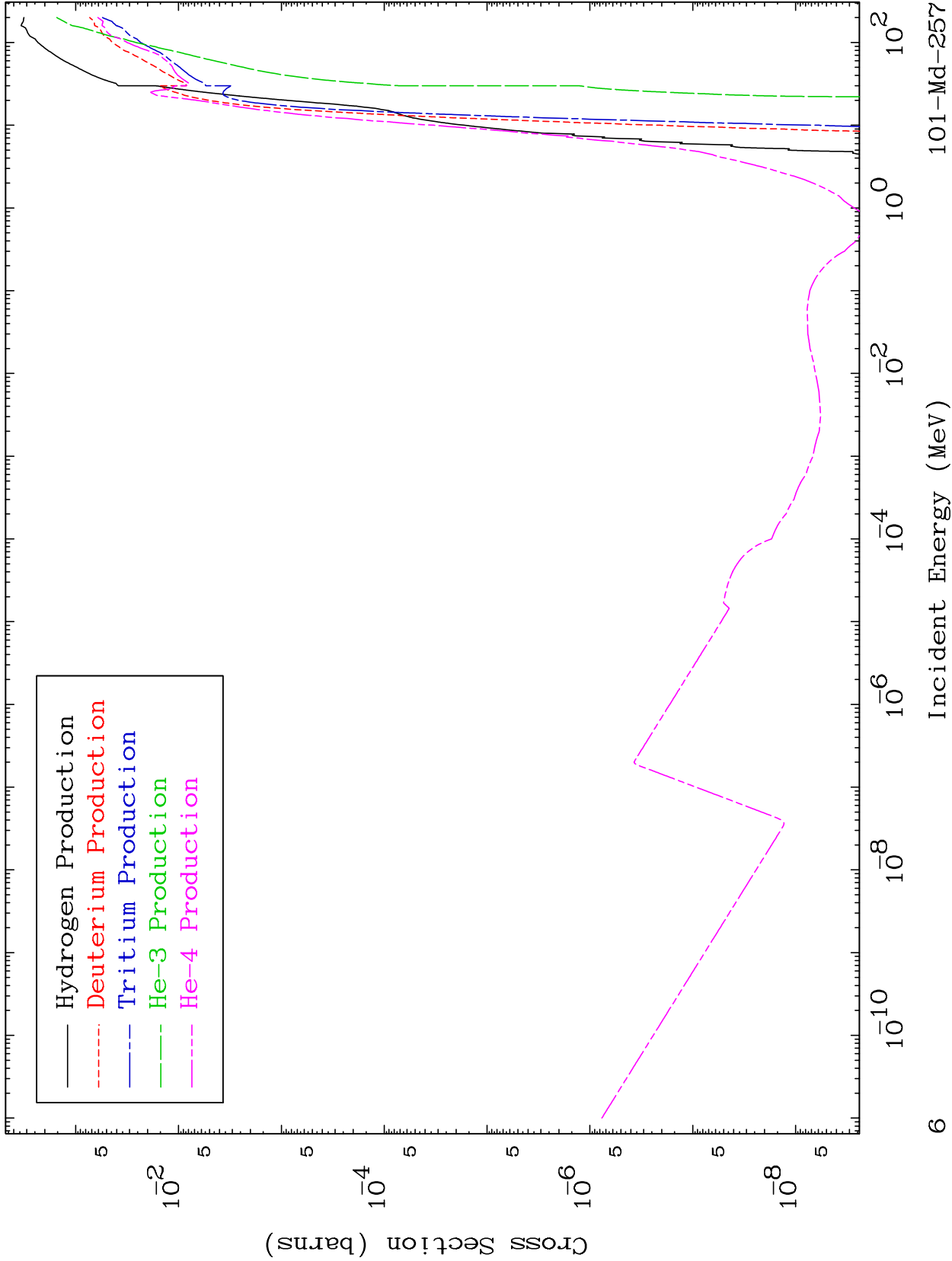


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Charged Particle
293 Kelvin Cross Sections

101-Md-257

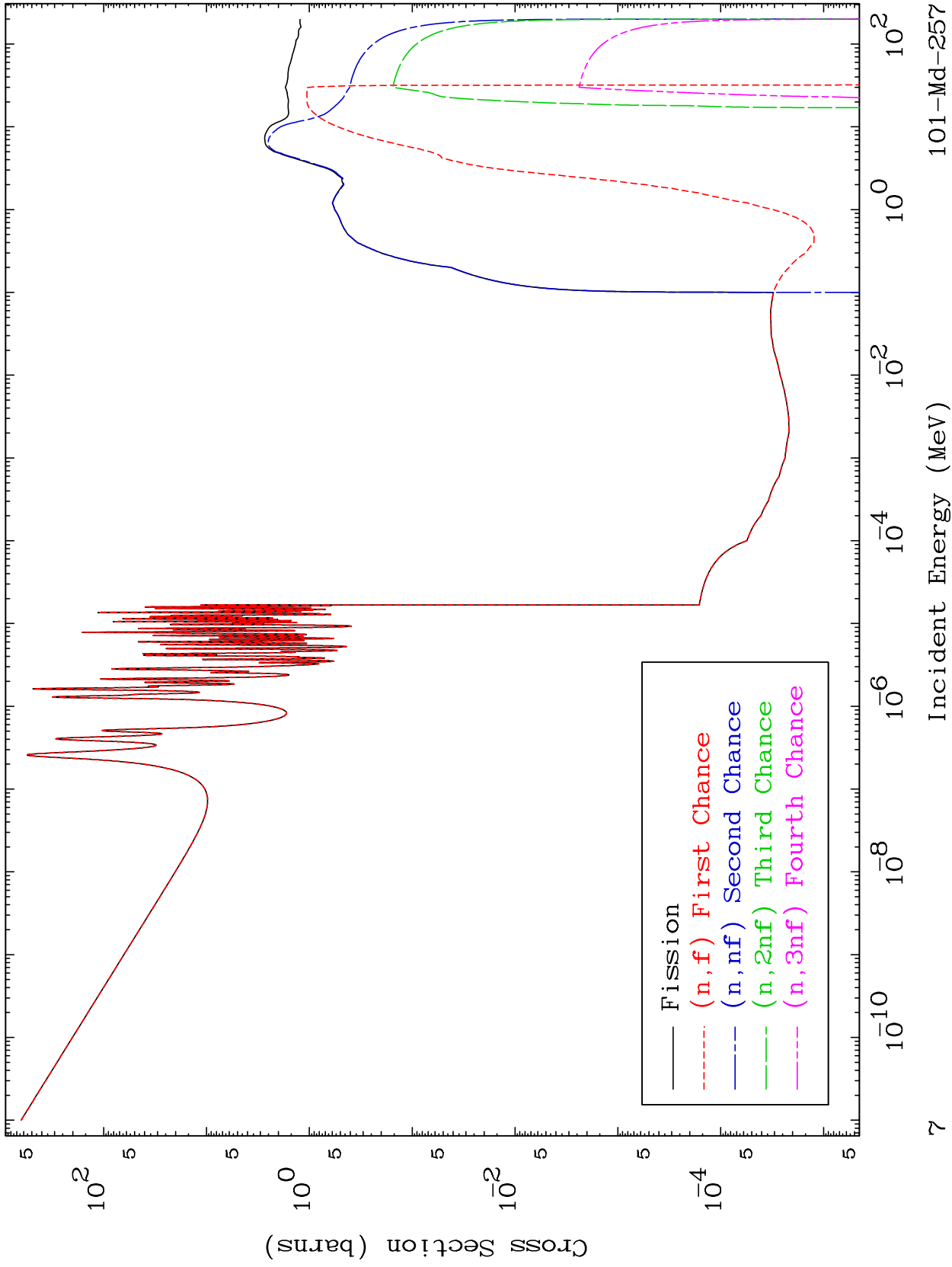




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Fission
293 Kelvin Cross Sections

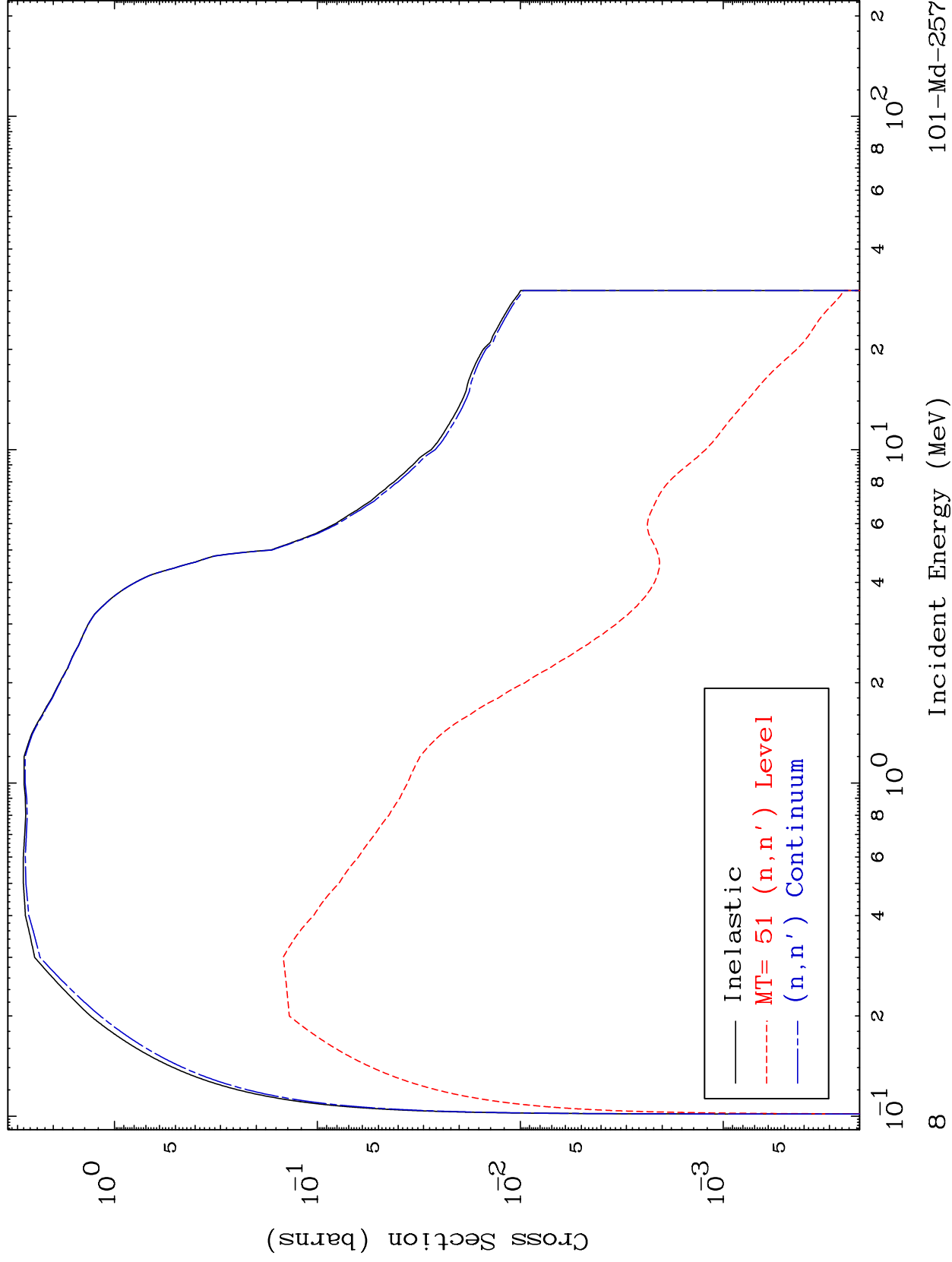
101-Md-257



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(n,n') Level
293 Kelvin Cross Sections

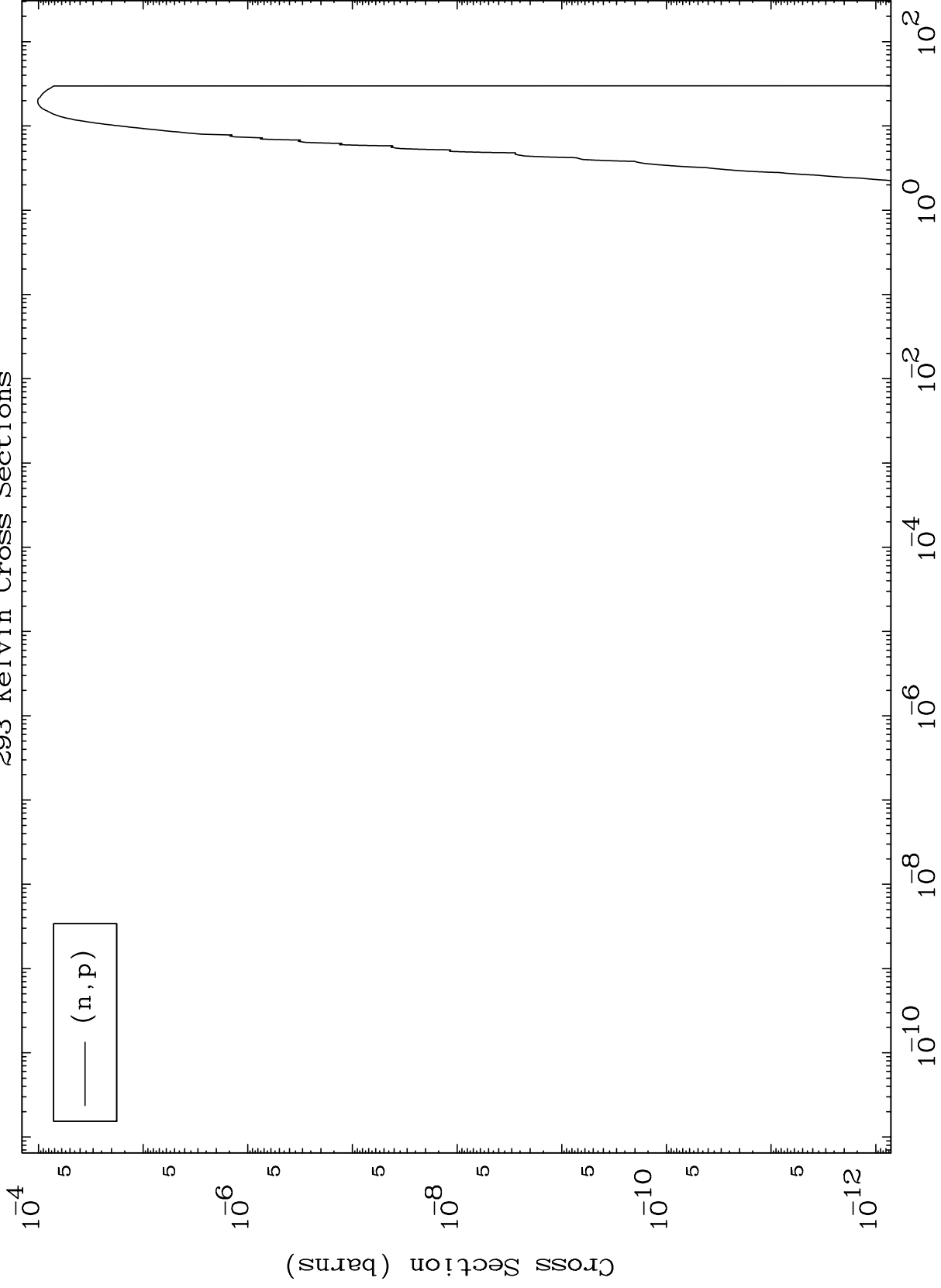
101-Md-257



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(n,p) Levels
293 Kelvin Cross Sections

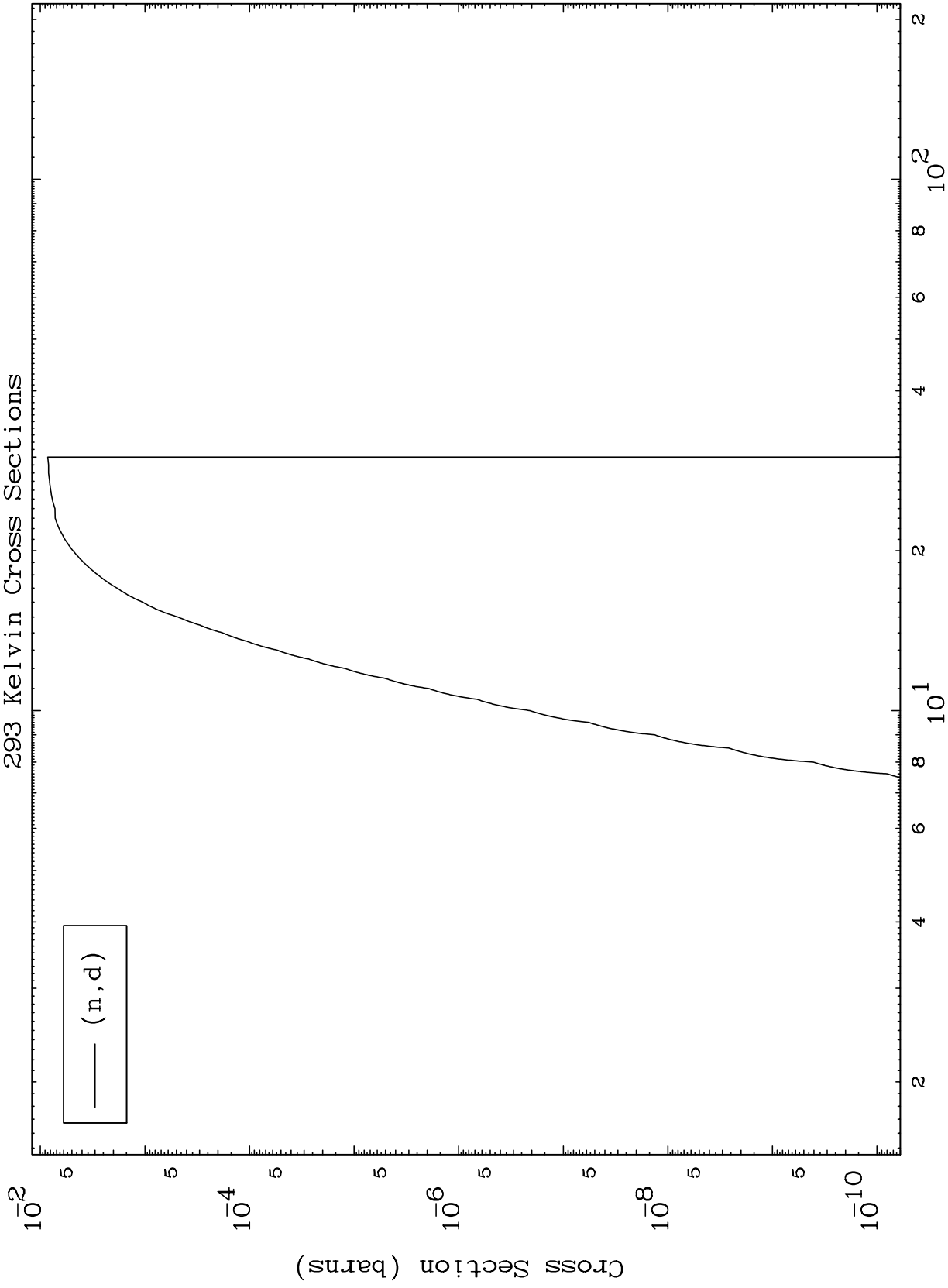
101-Md-257



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-257



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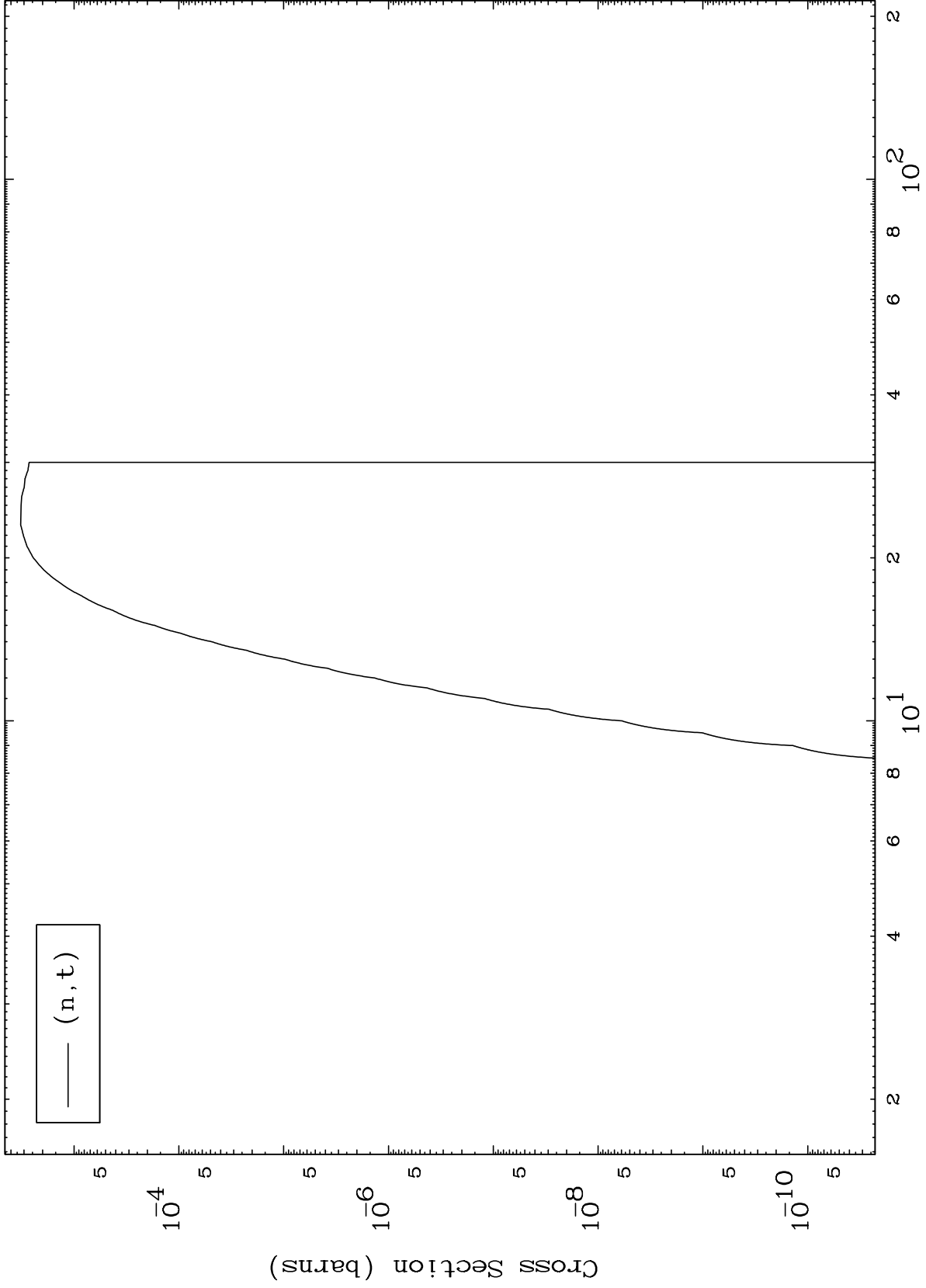
Incident Energy (MeV)

101-Md-257

MAT 9958

(n,t) Levels
293 Kelvin Cross Sections

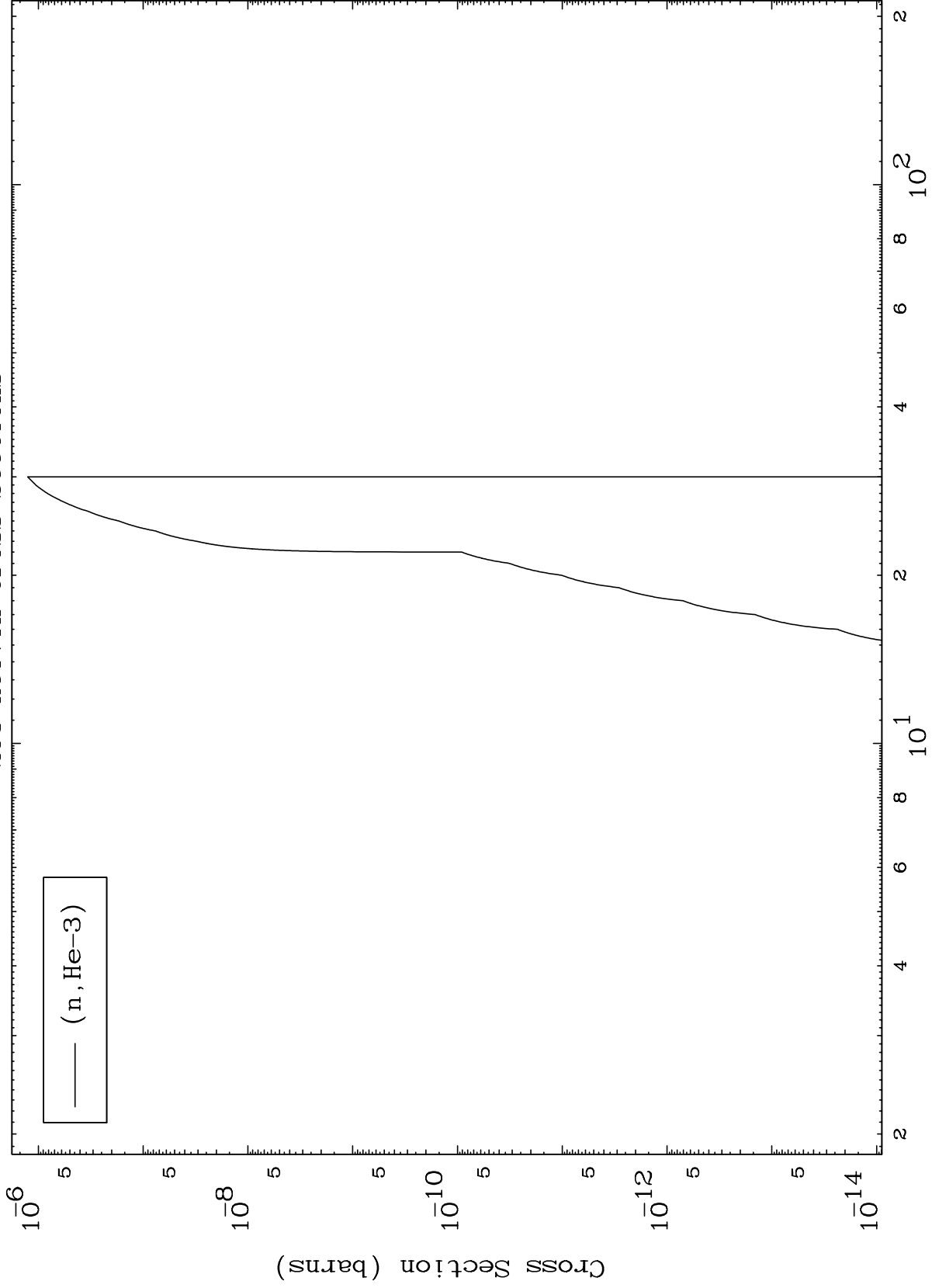
101-Md-257



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(n,He3) Levels
293 Kelvin Cross Sections

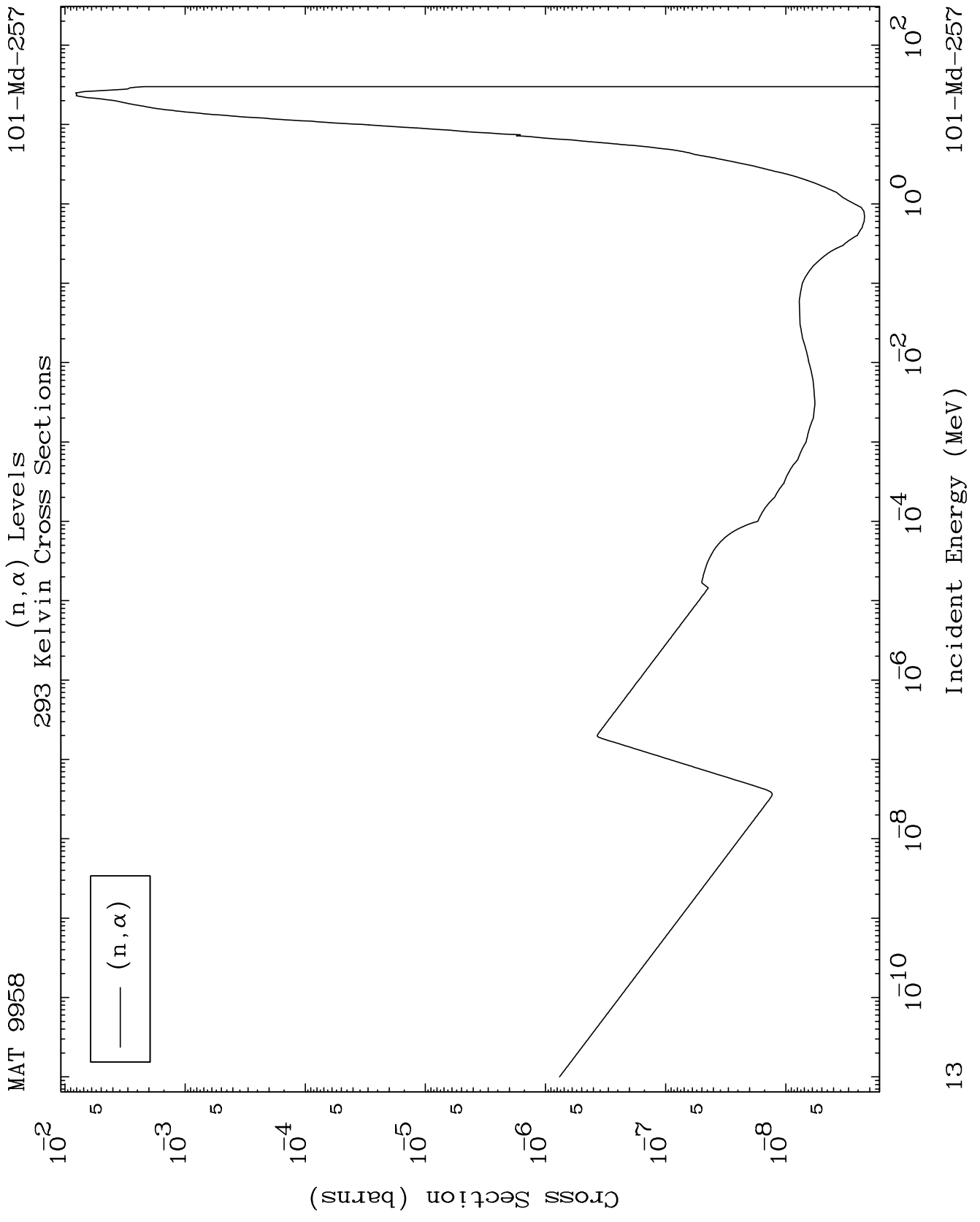
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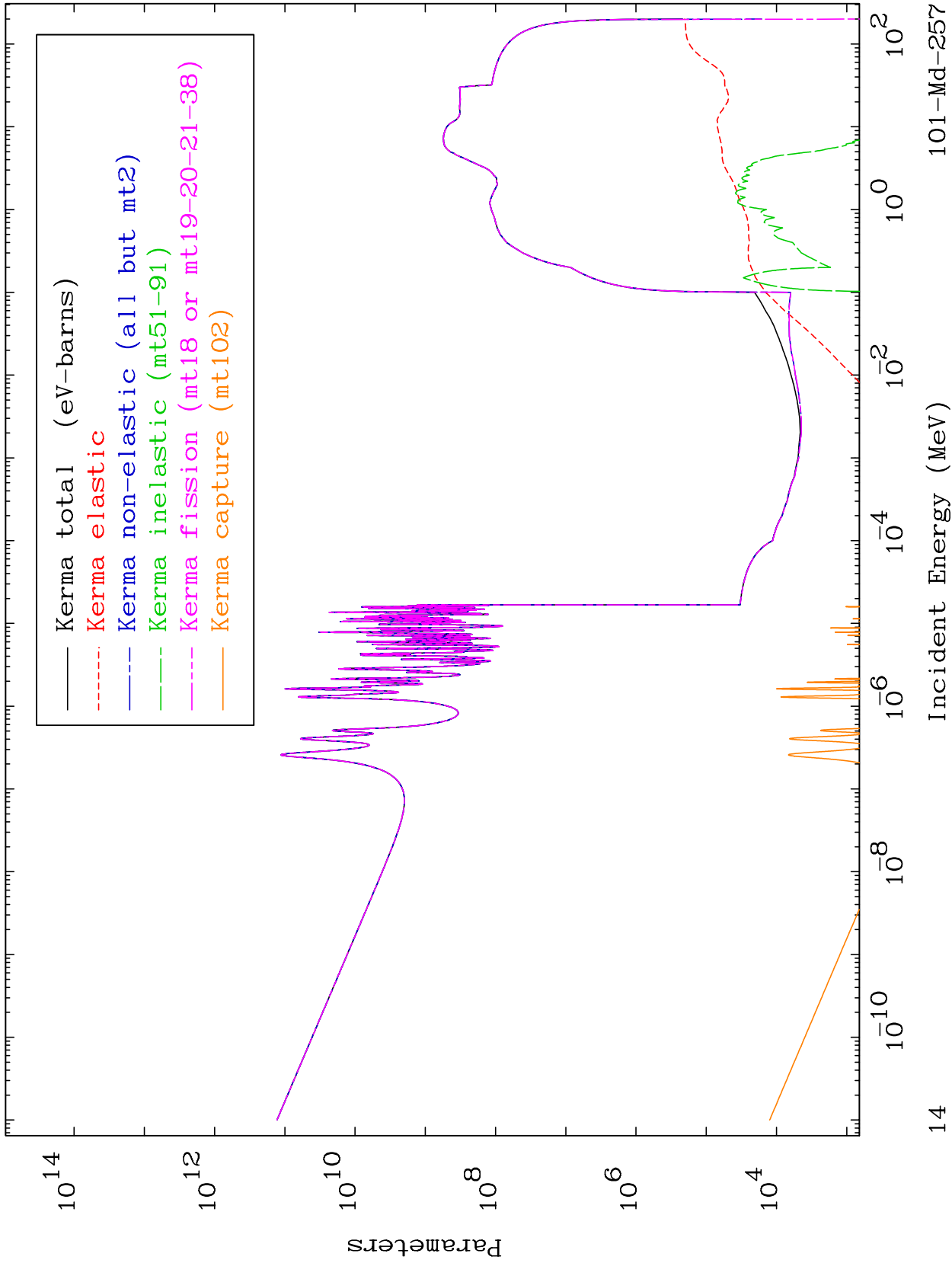


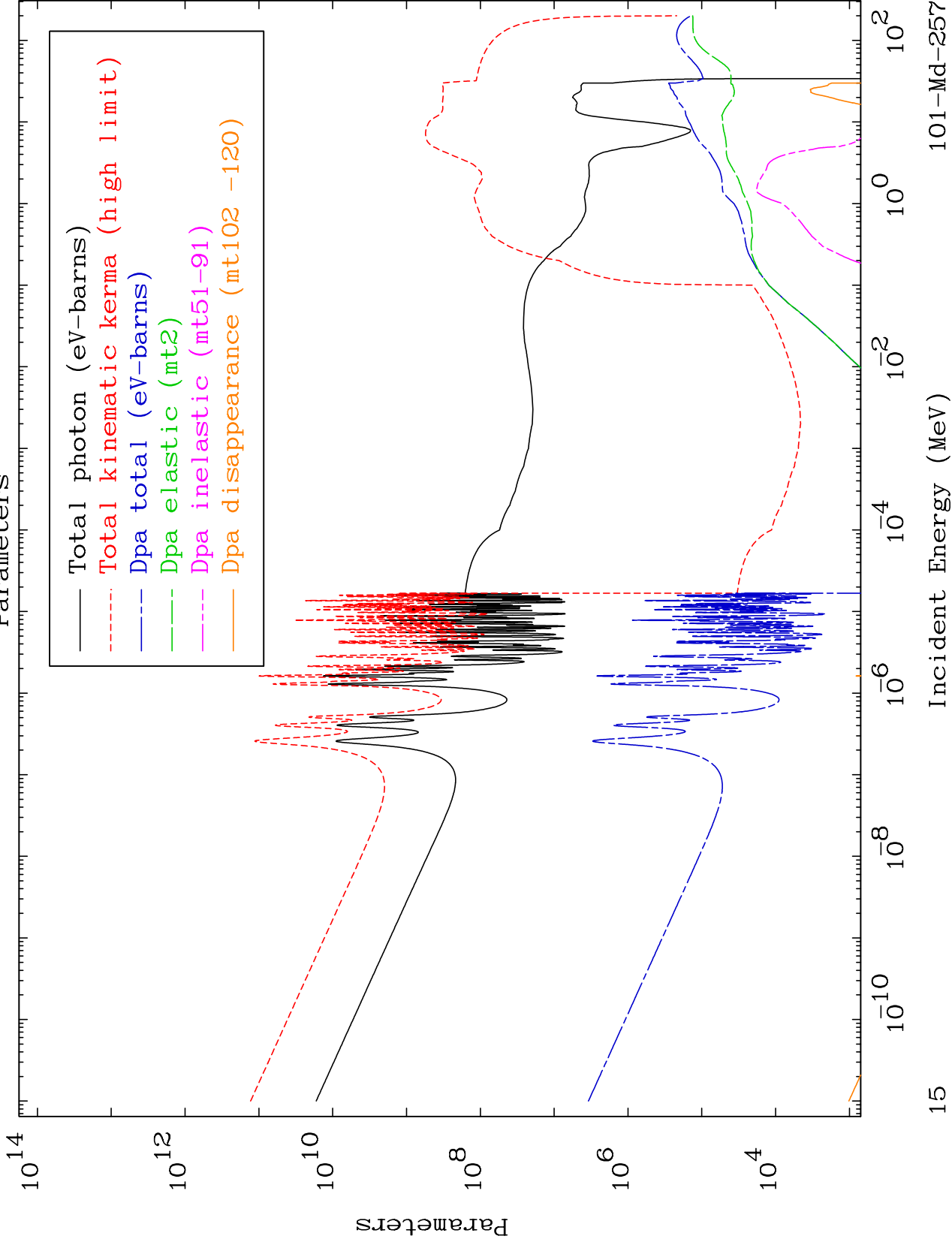
12

Incident Energy (MeV)

101-Md-257



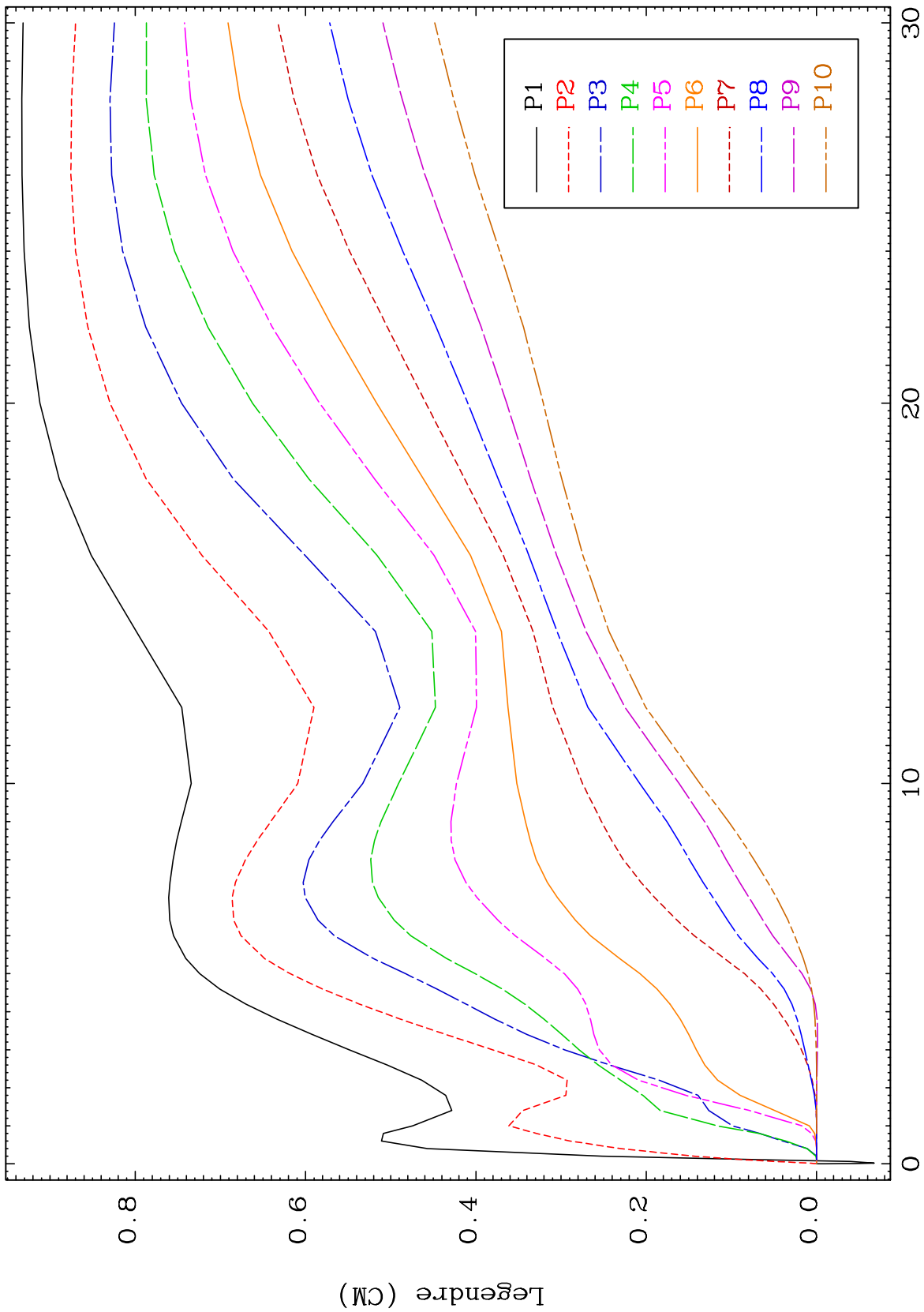




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101-Md-257

Elastic Legendre Coefficients



101-Md-257

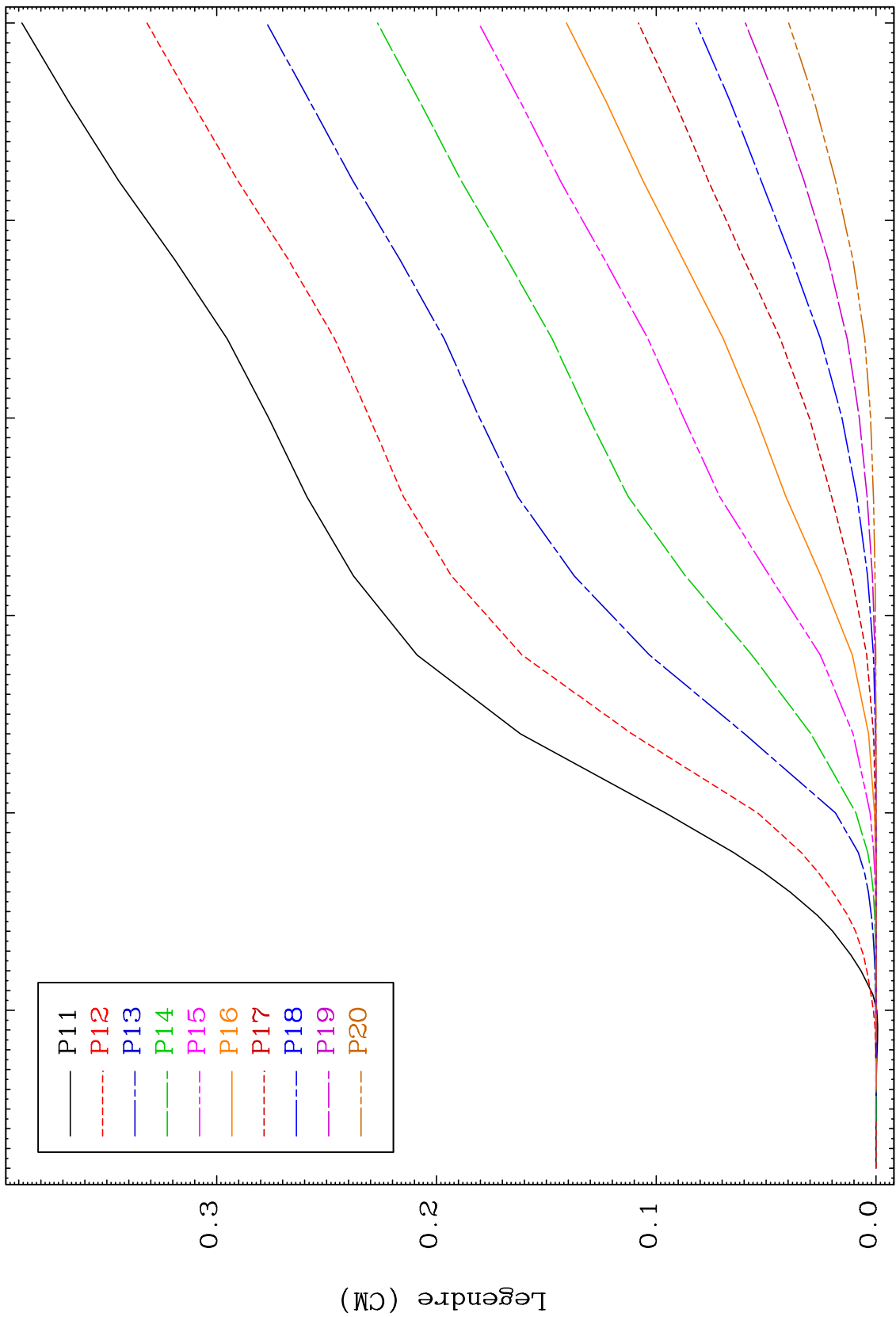
Incident Energy (MeV)

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Elastic Legendre Coefficients

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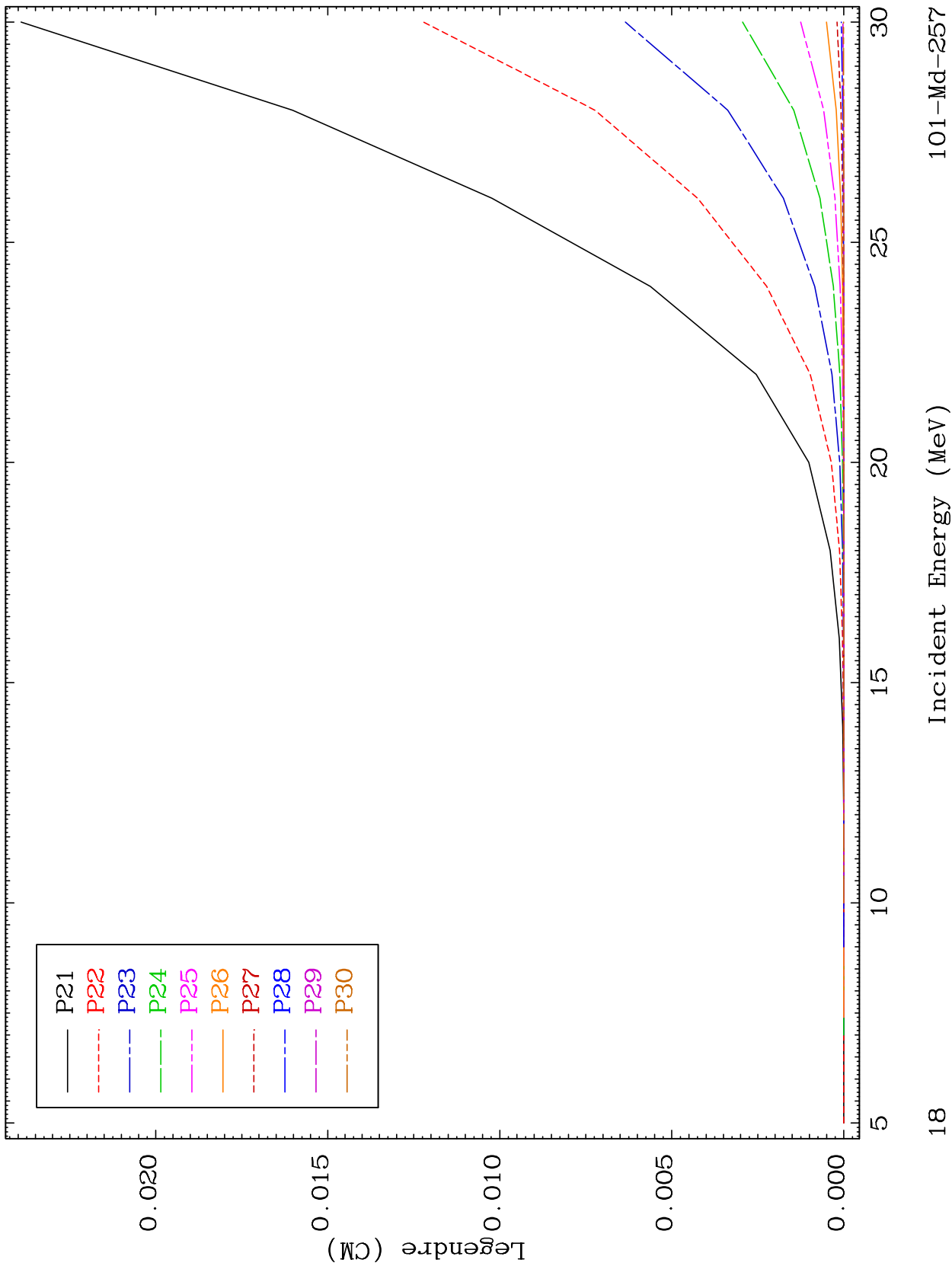
Incident Energy (MeV)

101-Md-257

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Elastic Legendre Coefficients

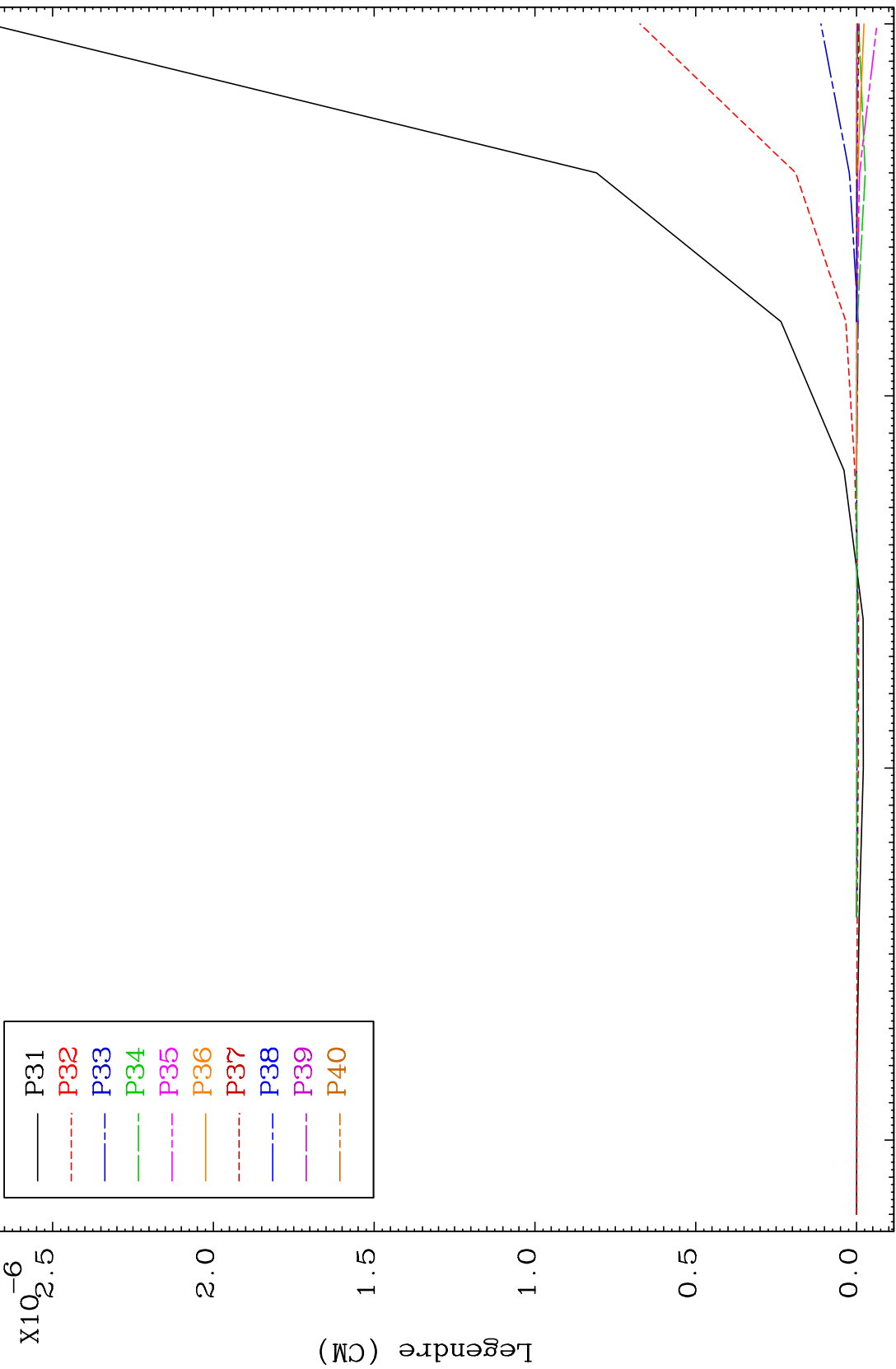
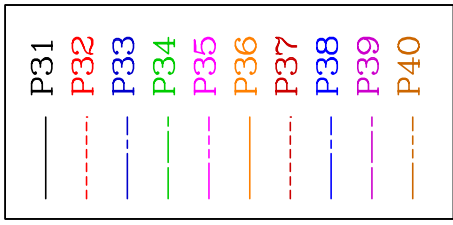
101-Md-257



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Elastic Legendre Coefficients

101-Md-257



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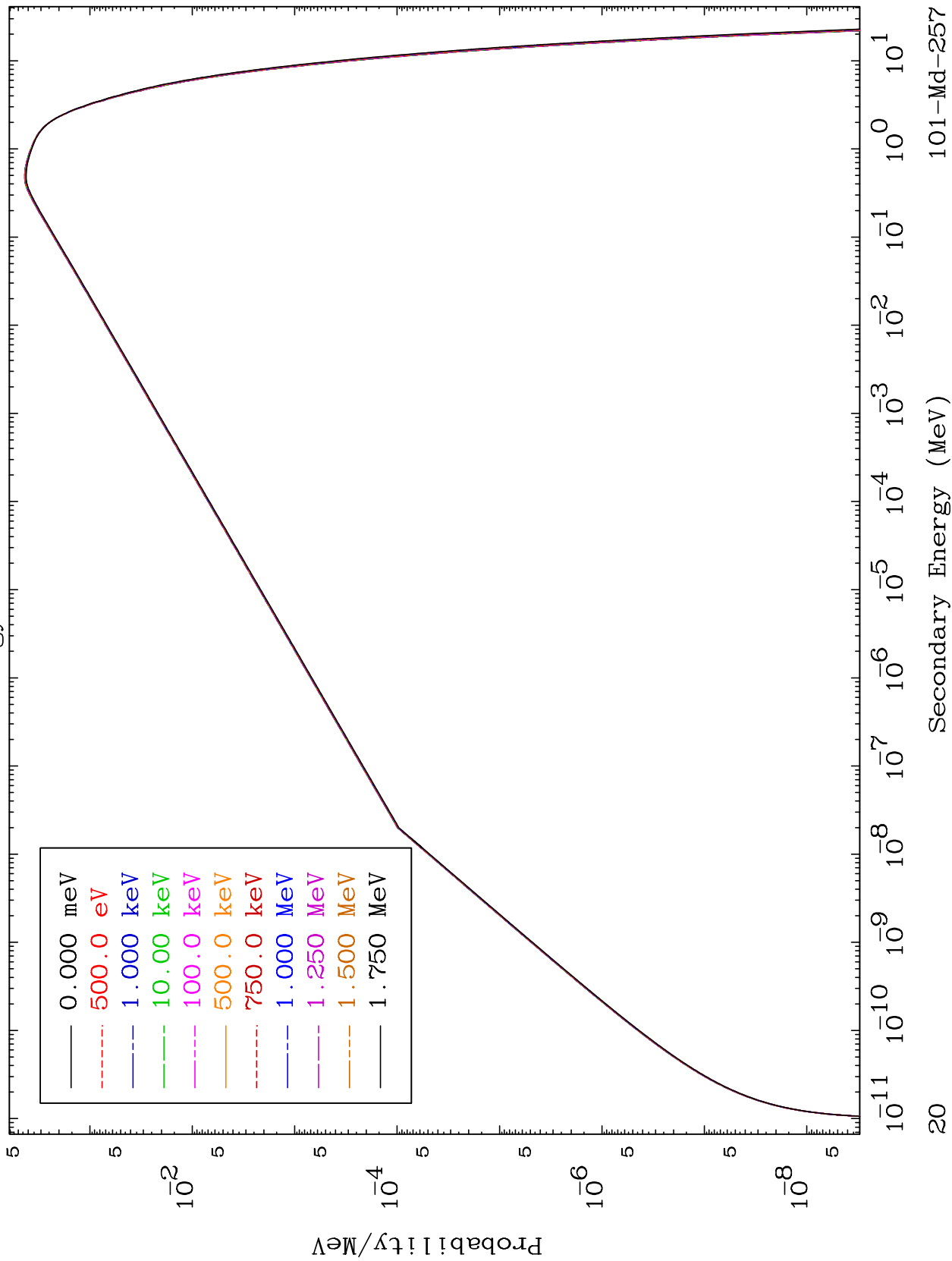
Incident Energy (MeV)

101-Md-257

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Fission Energy Distribution

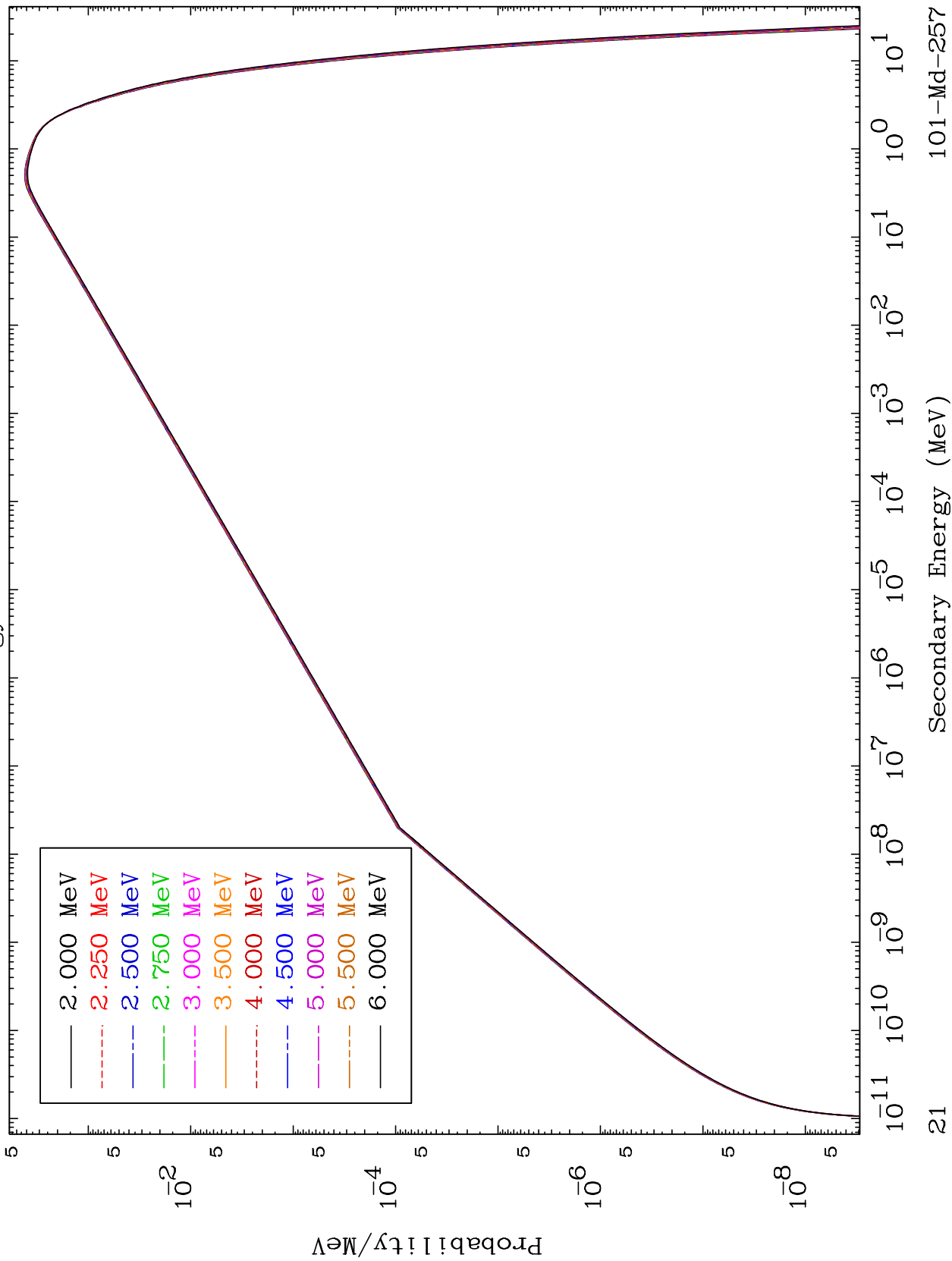
101-Md-257



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Fission Energy Distribution

101-Md-257



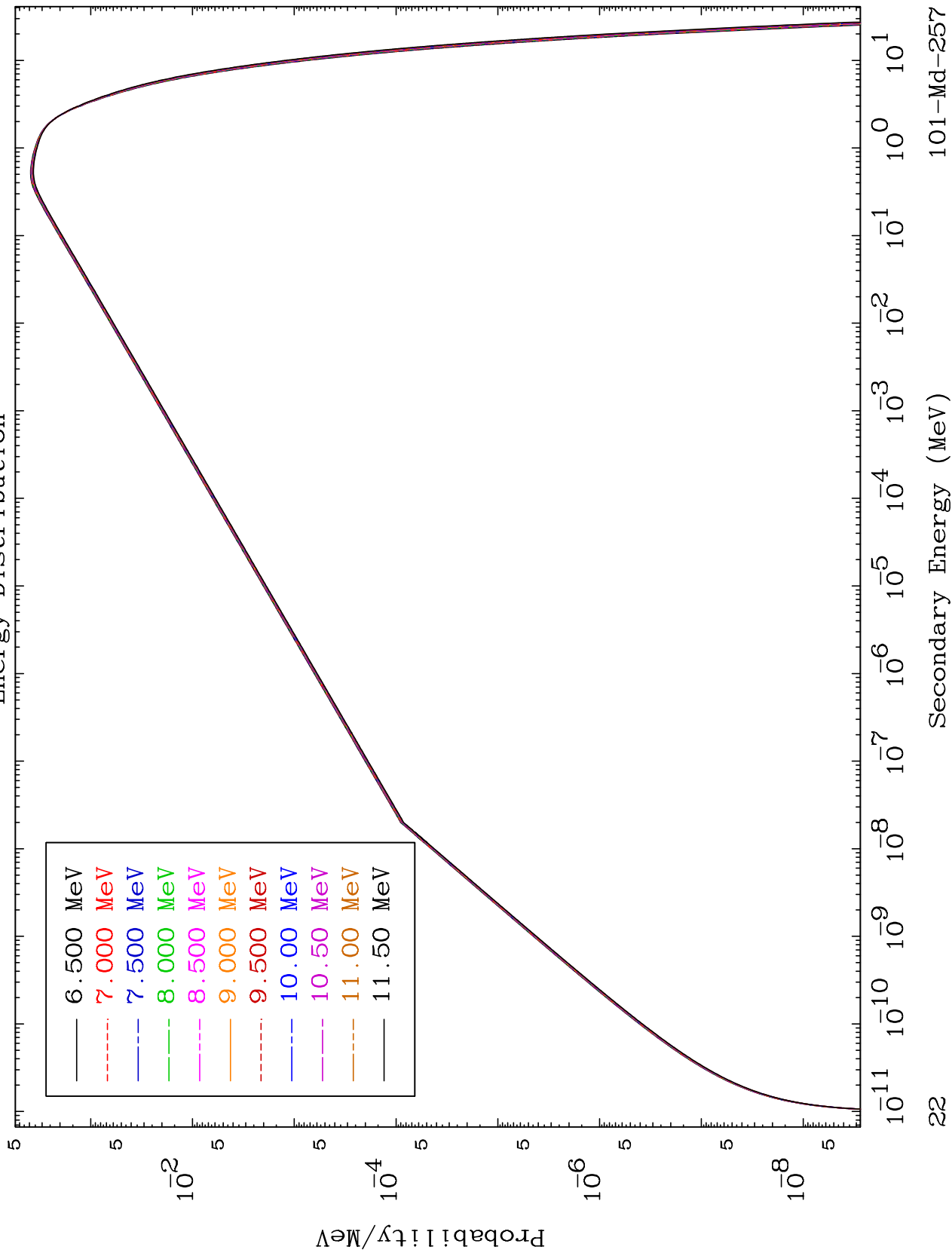
101-Md-257

Secondary Energy (MeV)

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Fission Energy Distribution

101-Md-257



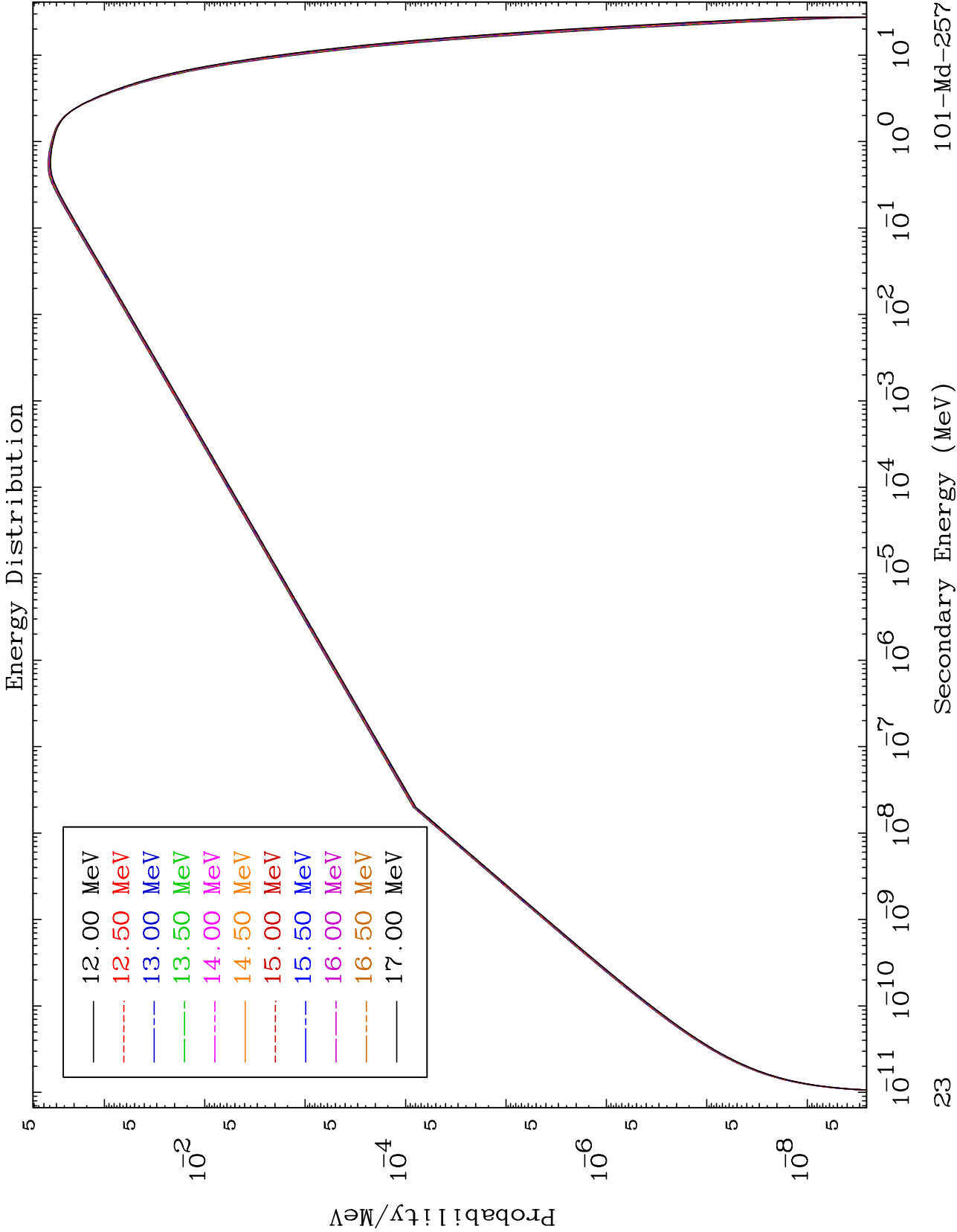
101-Md-257

Secondary Energy (MeV)

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Fission

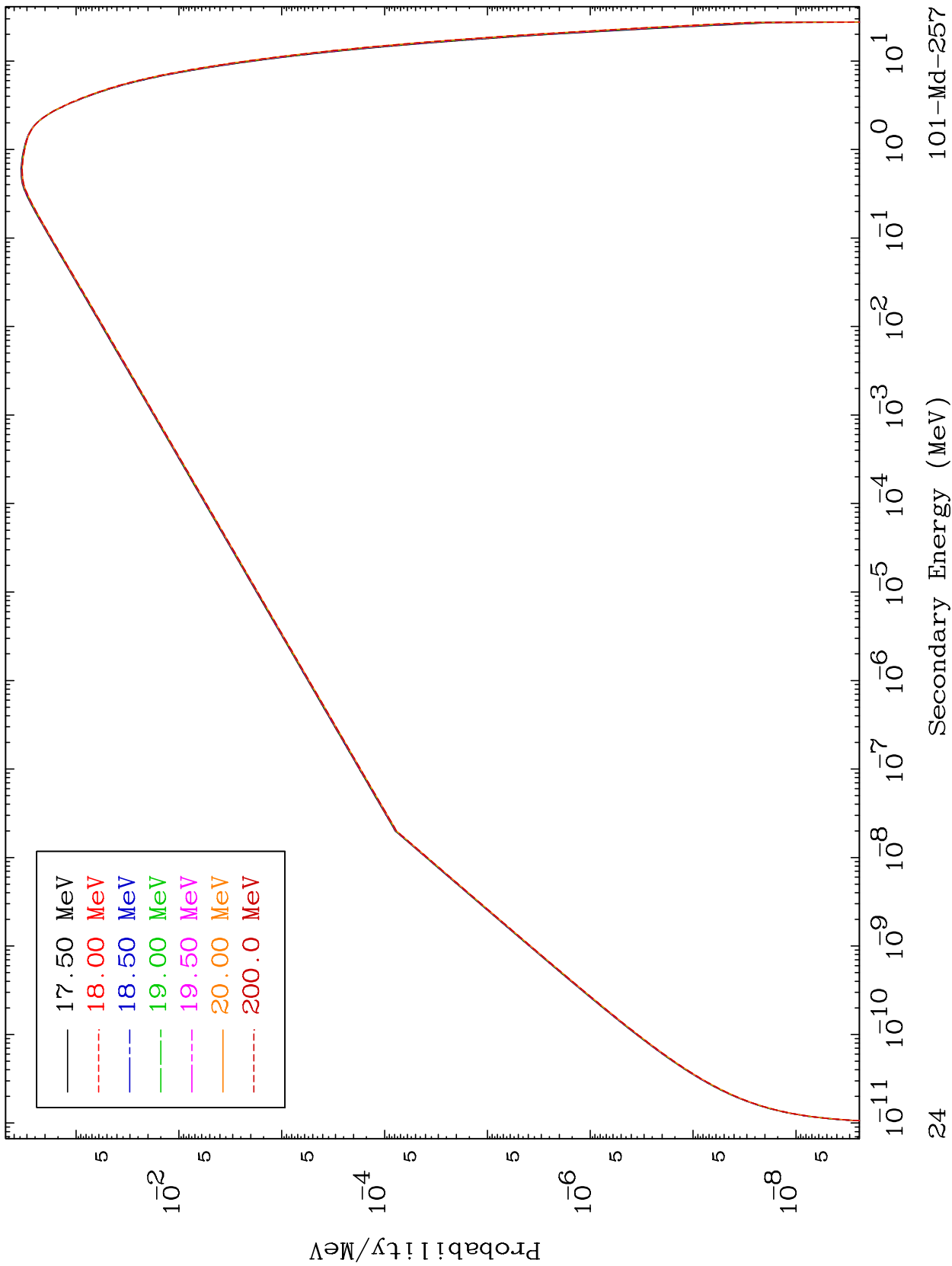
101-Md-257



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Fission Energy Distribution

101-Md-257



101-Md-257

Secondary Energy (MeV)

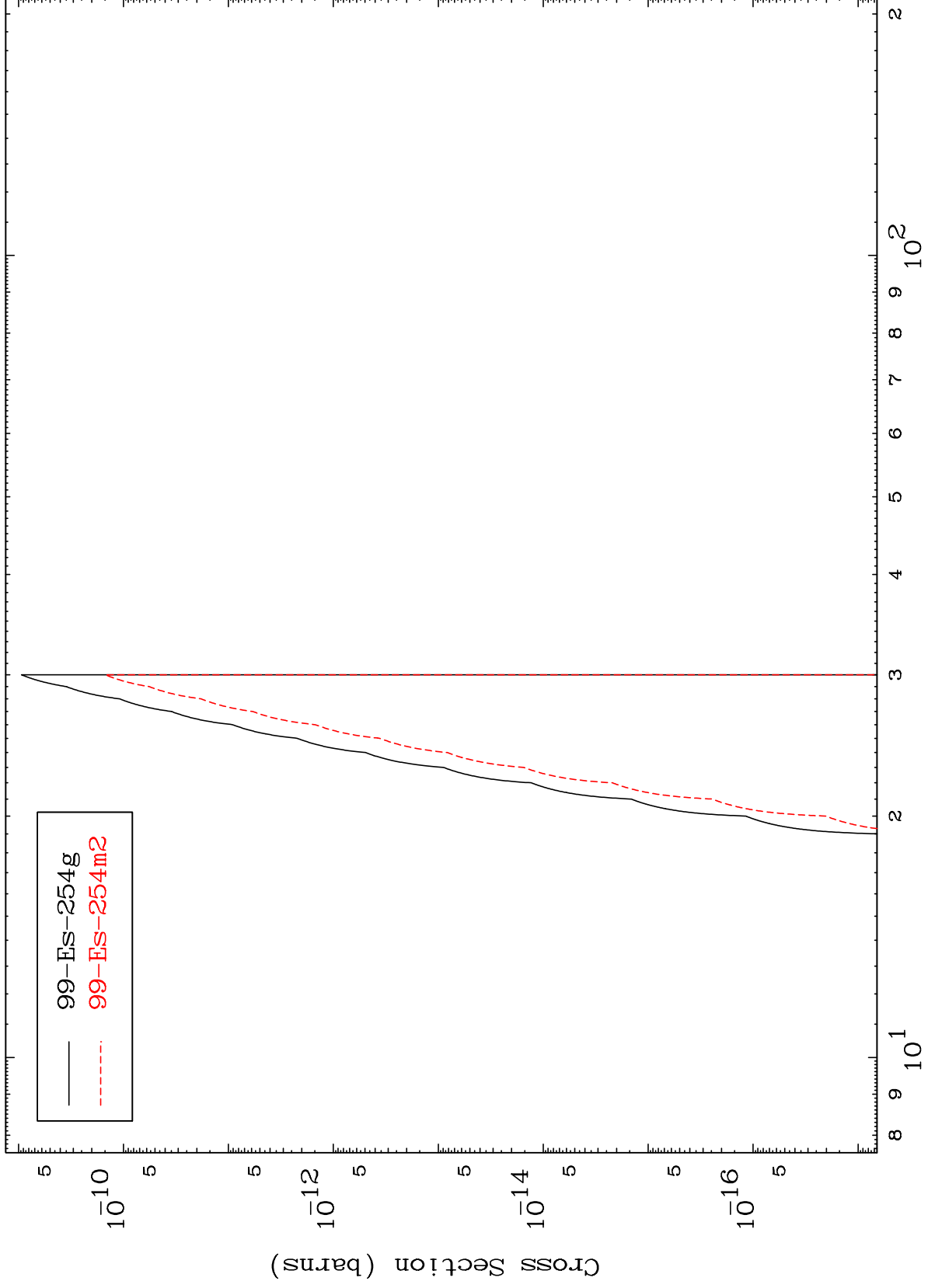
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(n,n') He-3

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

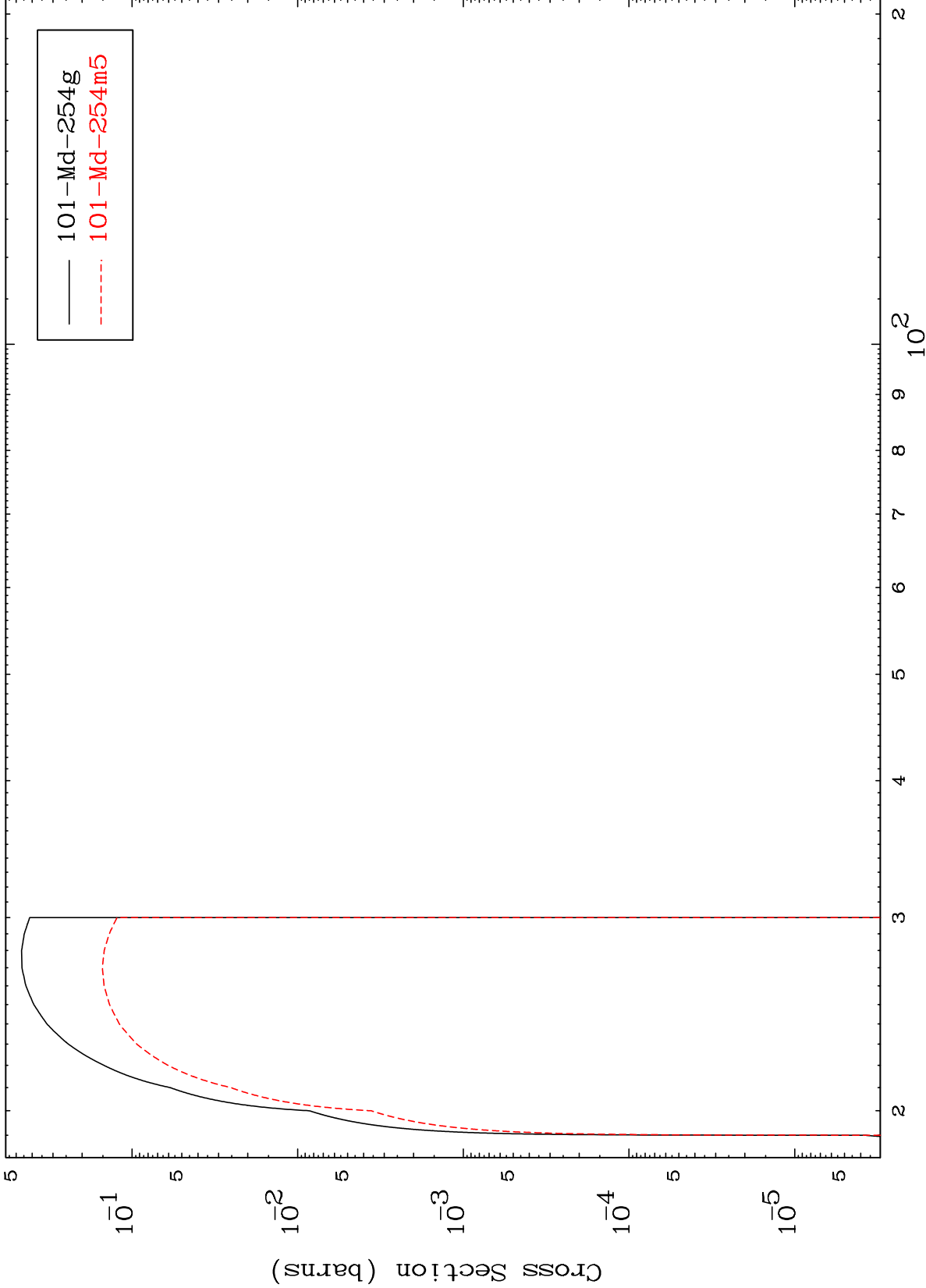
101-Md-257

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(n,4n)

101-Md-257

Radionuclide Production Cross Section



Incident Energy (MeV)

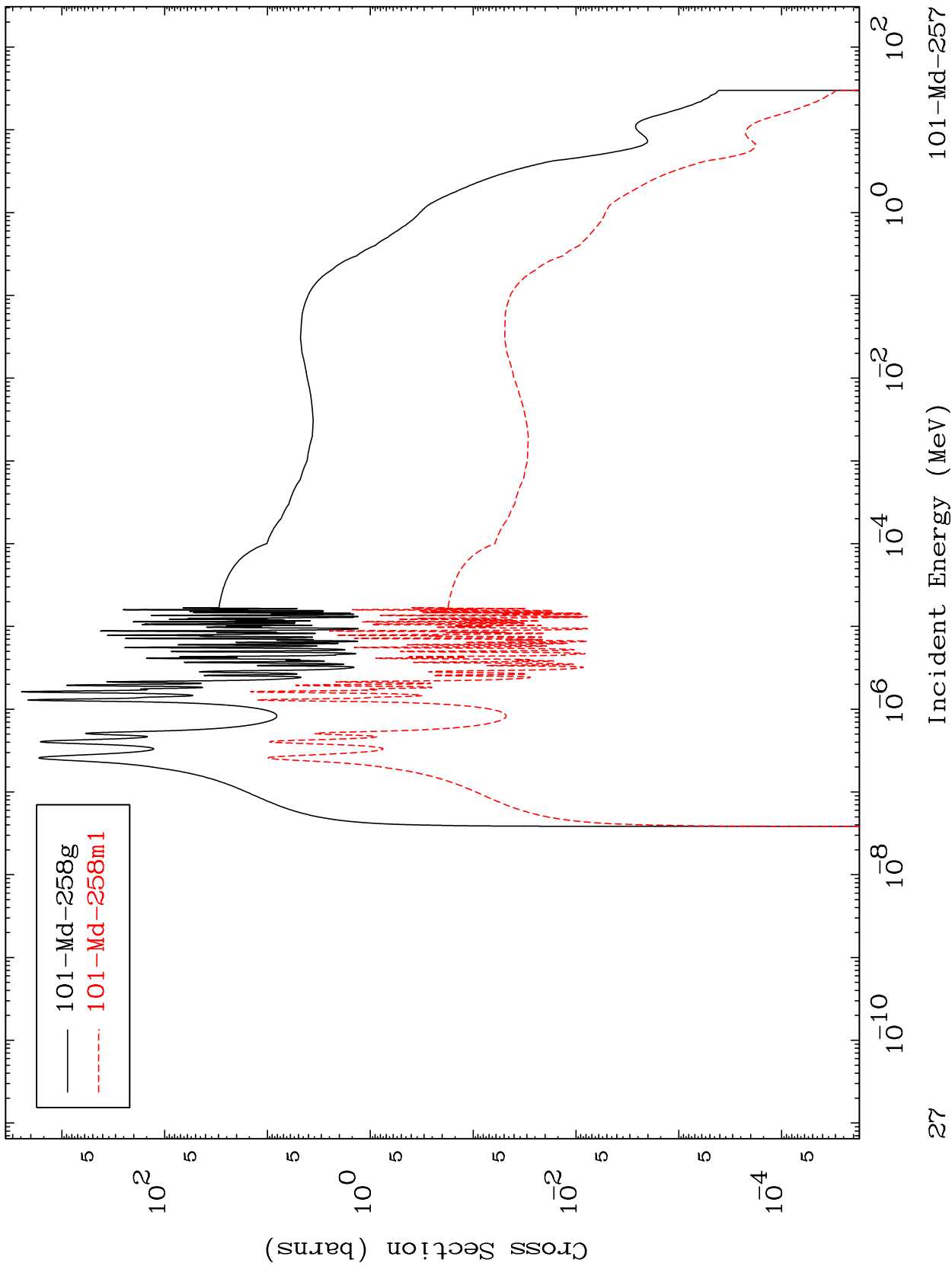
101-Md-257

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101-Md-257

(n, γ)
Radionuclide Production Cross Section

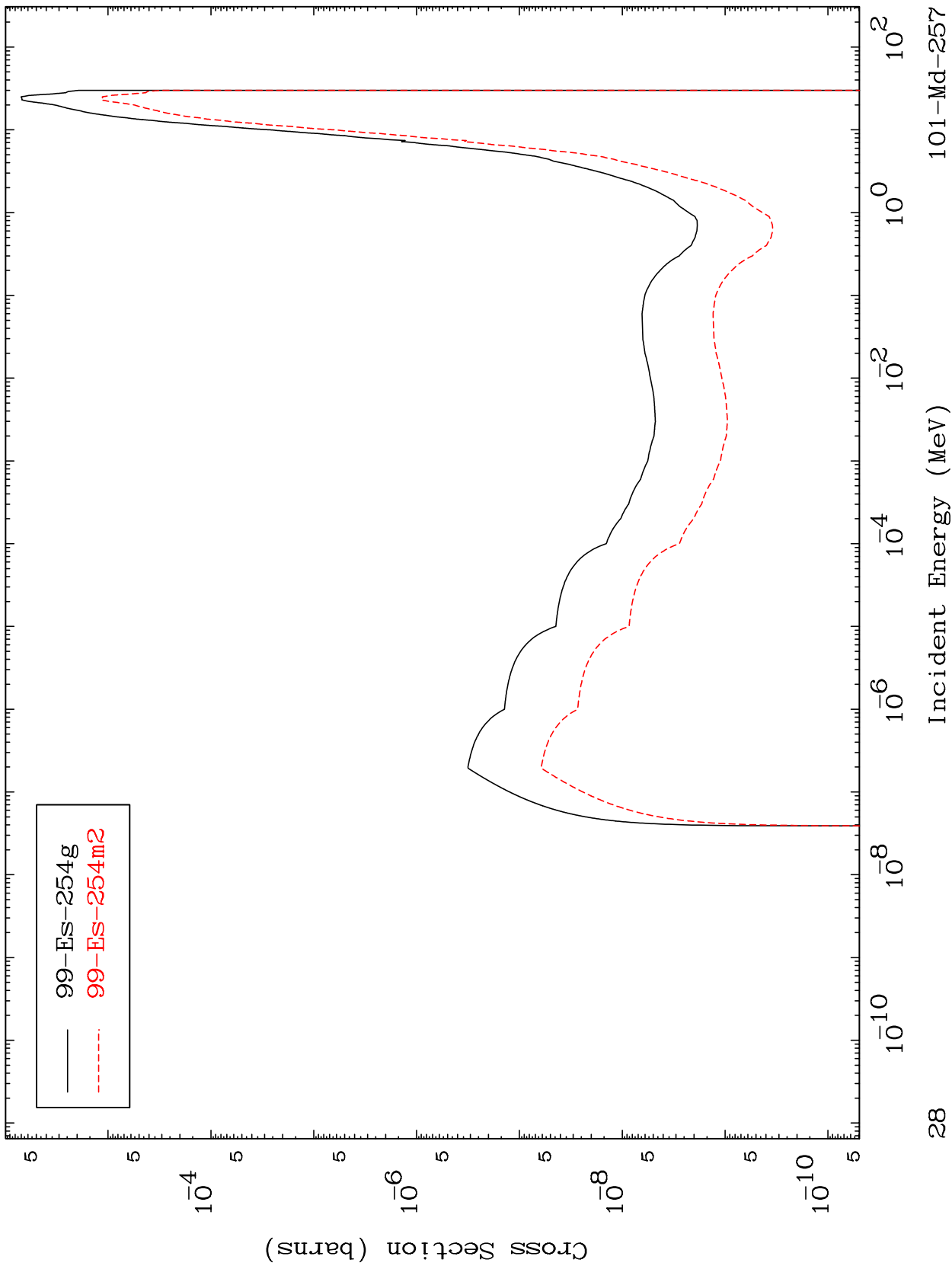


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(n, α)

101-Md-257

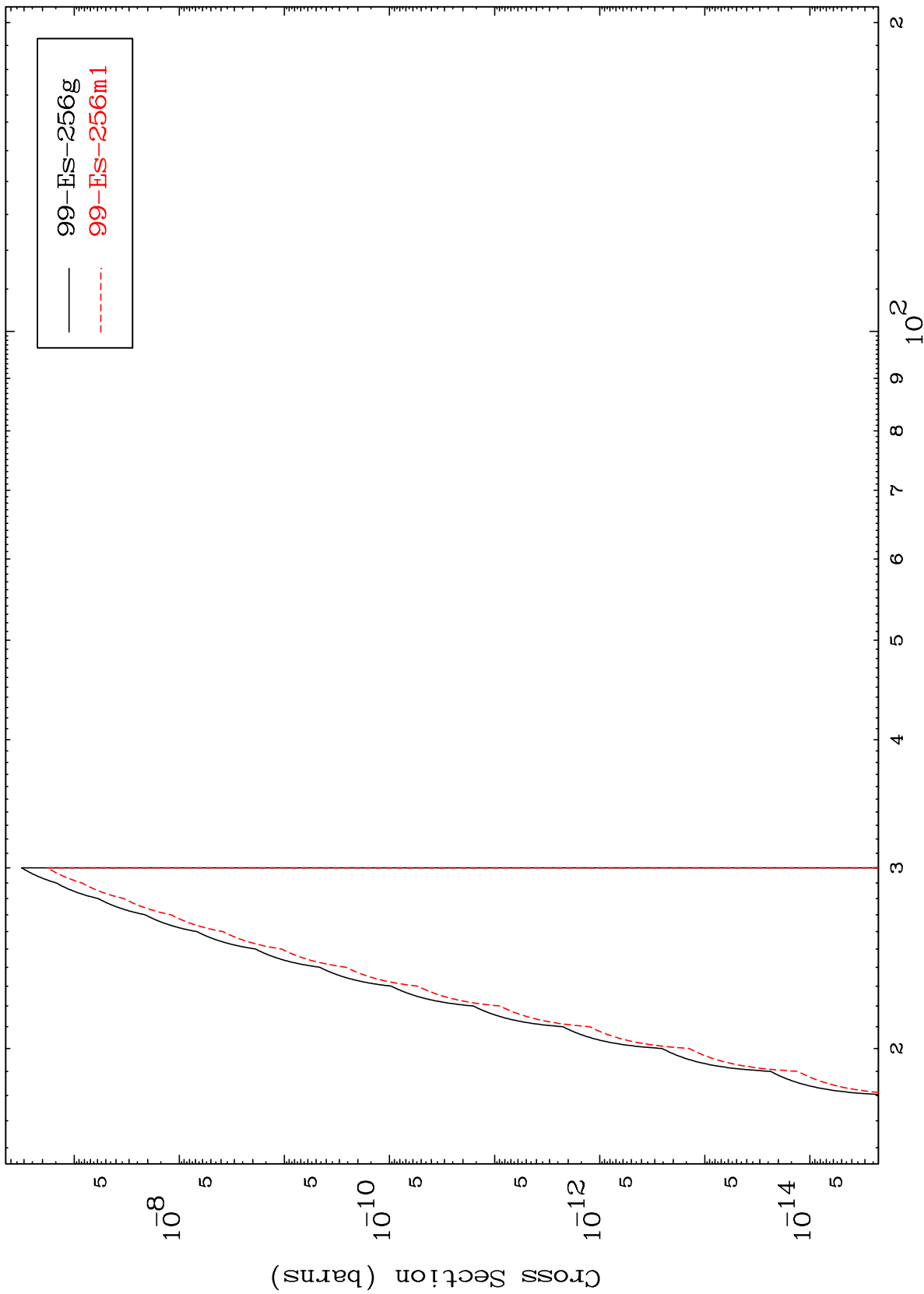
Radionuclide Production Cross Section



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101-Md-257

(n,2p)
Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-257