

Program EVALPLOT
(Version 2018-1)

by

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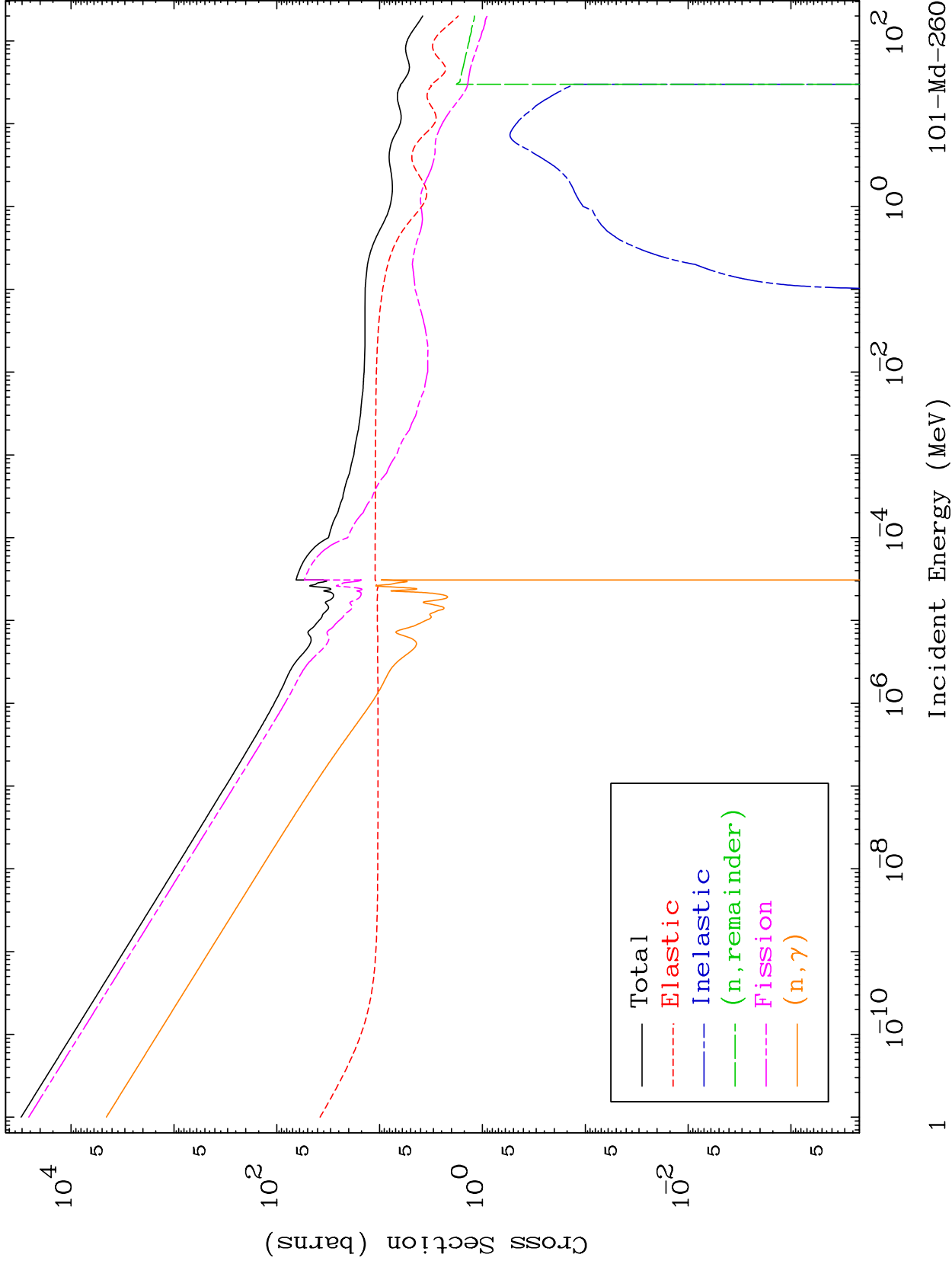
Press Mouse Button to Start

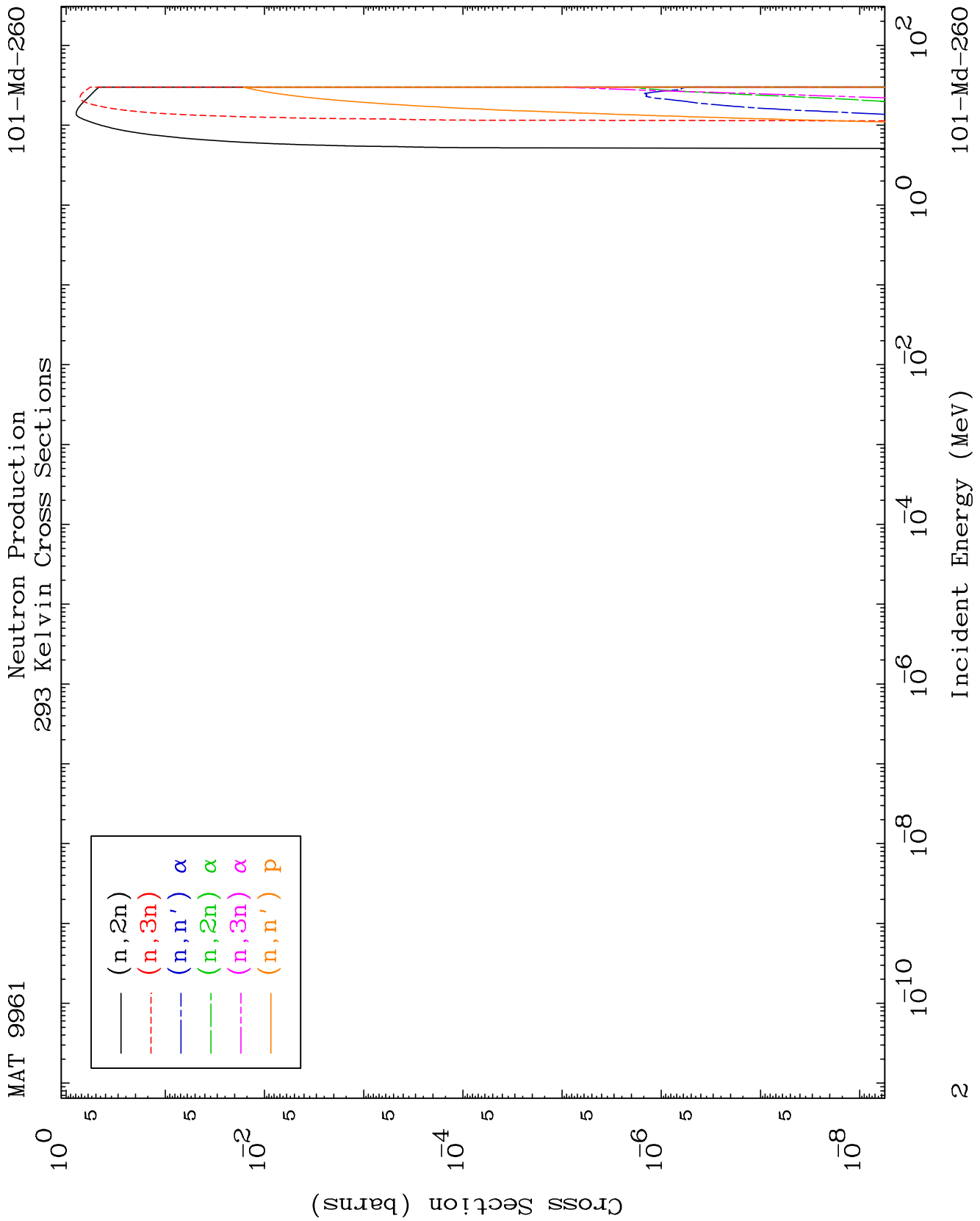
MAT 9961

Major

293 Kelvin Cross Sections

101-Md-260

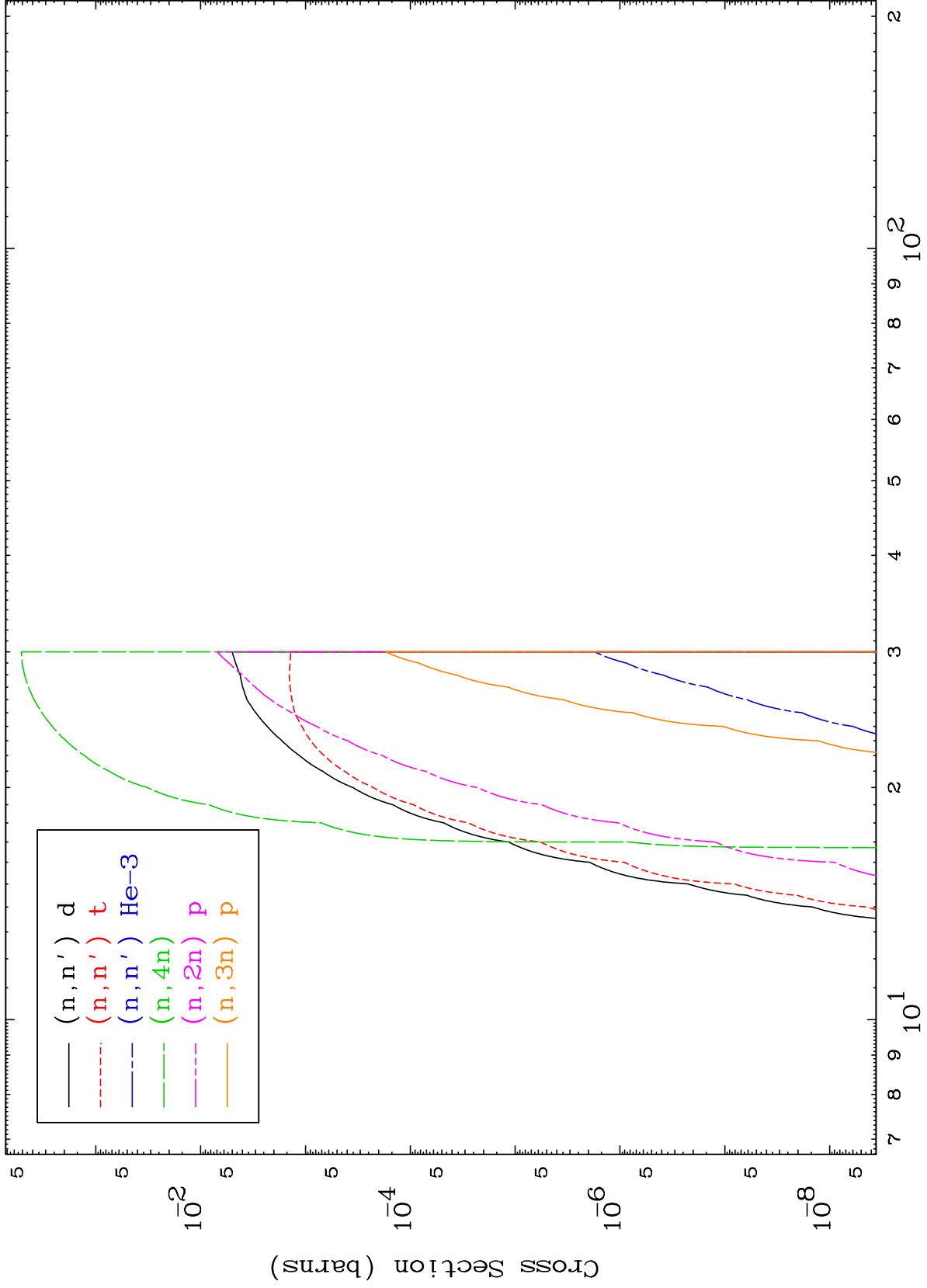




MAT 9961

Neutron Production
293 Kelvin Cross Sections

101-Md-260



3

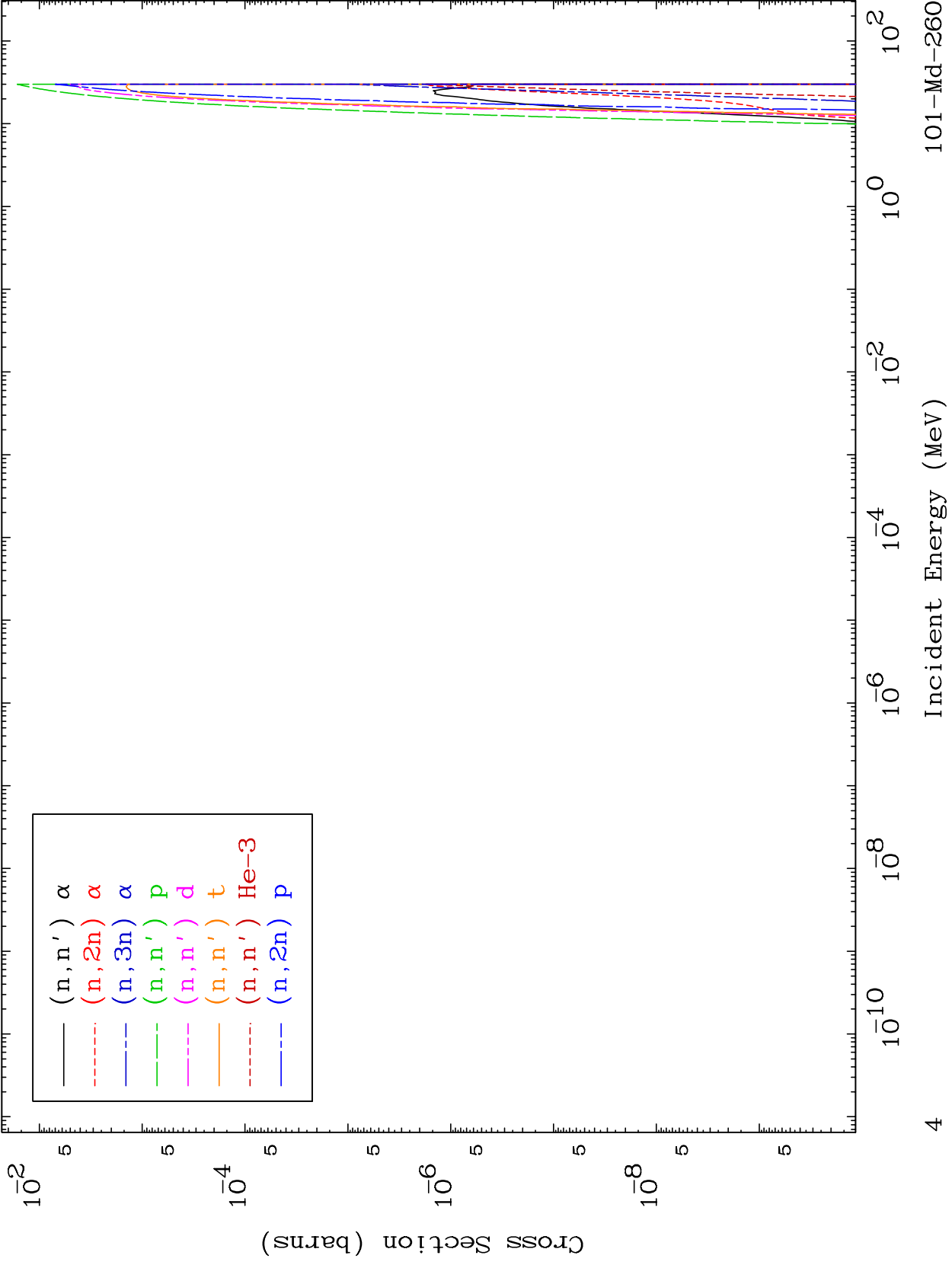
Incident Energy (MeV)

101-Md-260

MAT 9961

Charged Particle
293 Kelvin Cross Sections

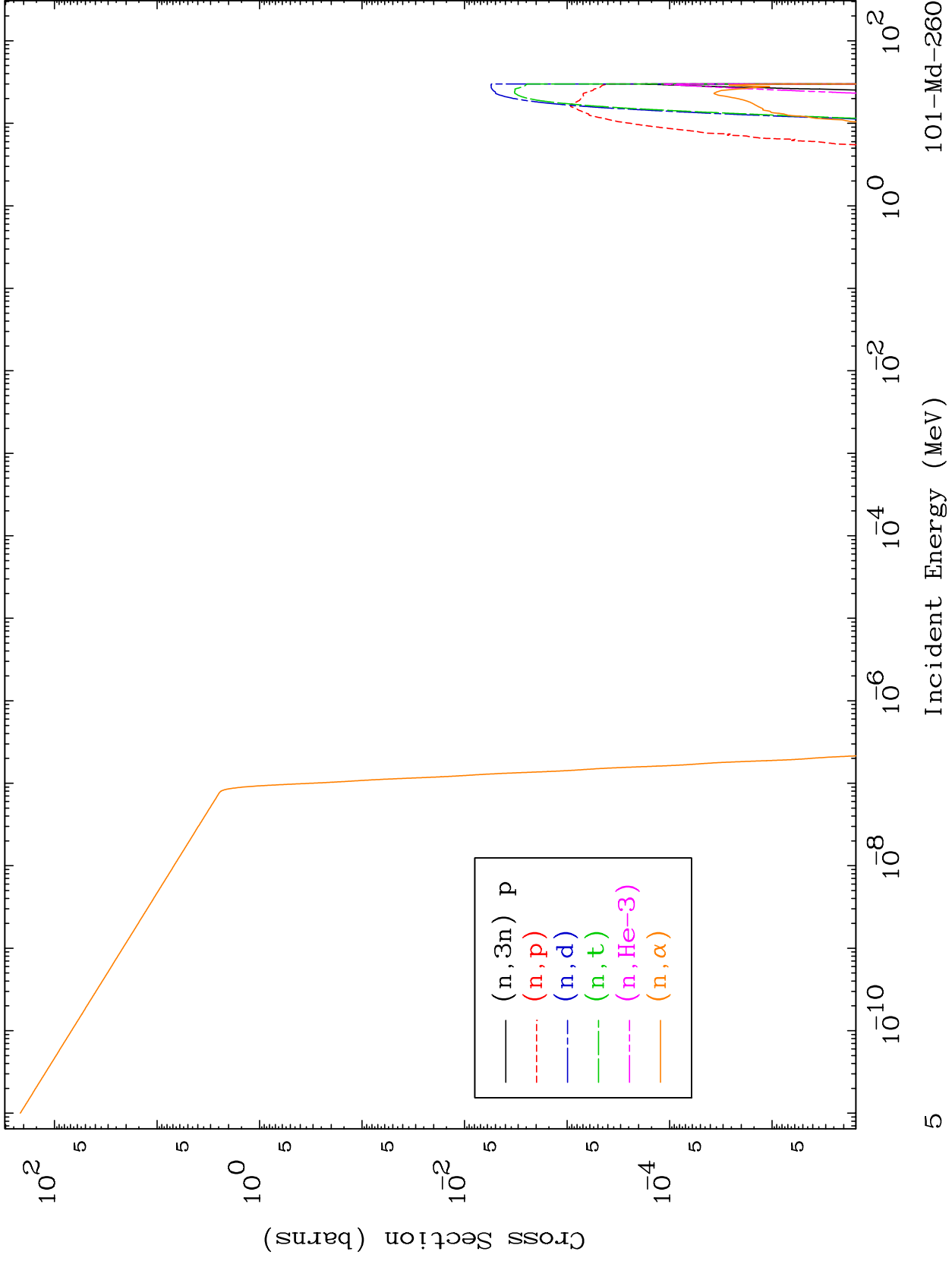
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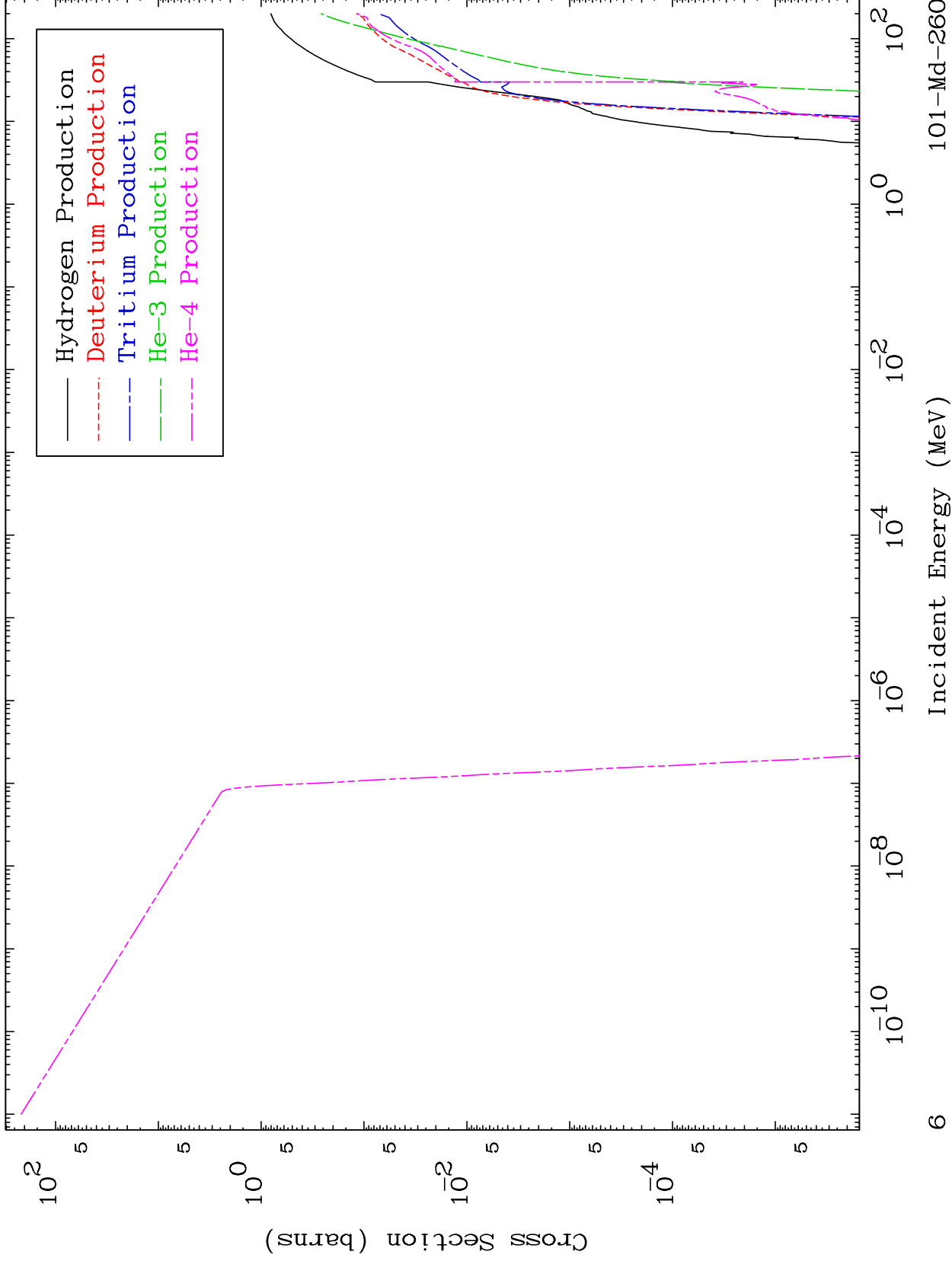


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Charged Particle
293 Kelvin Cross Sections

101-Md-260

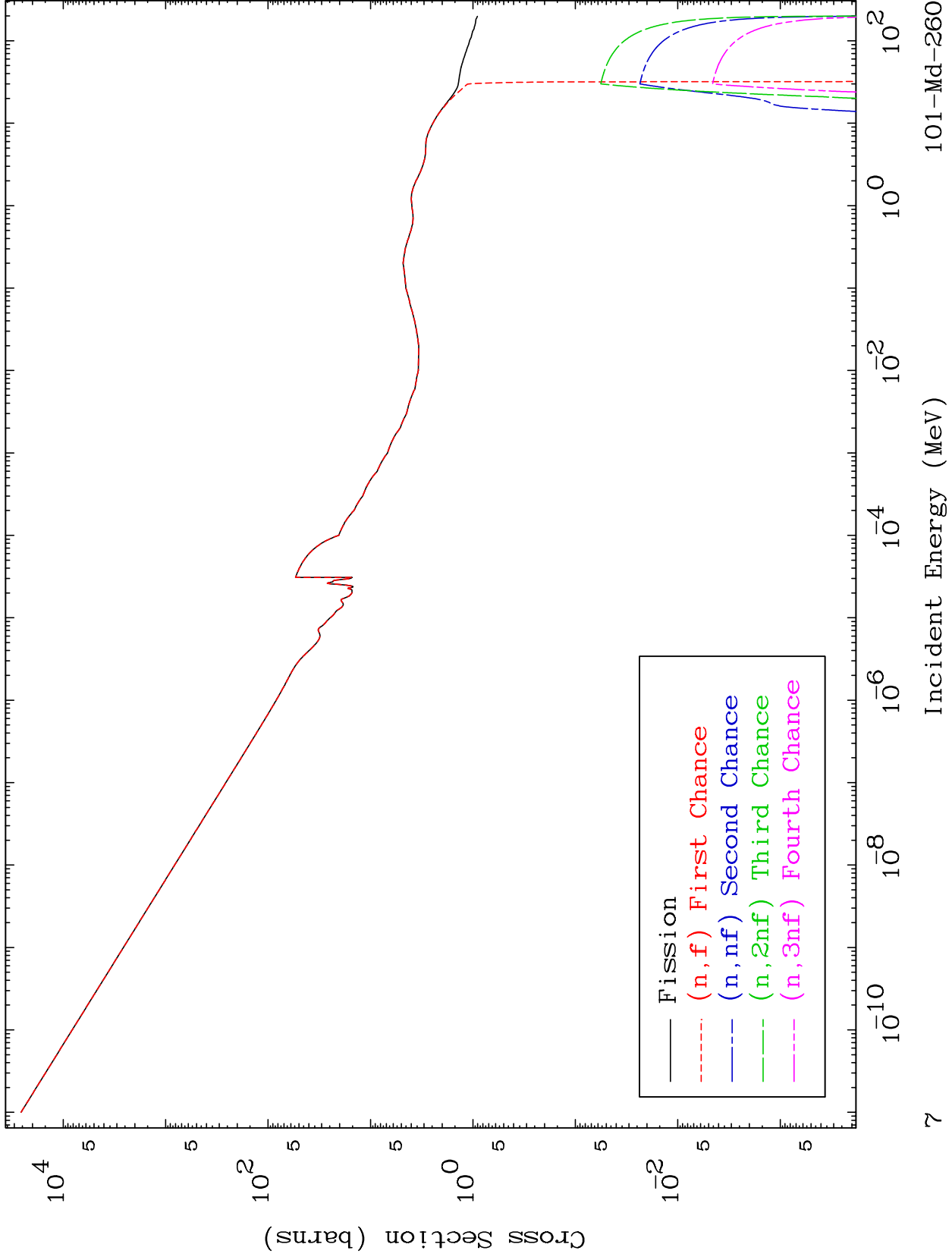




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Fission
293 Kelvin Cross Sections

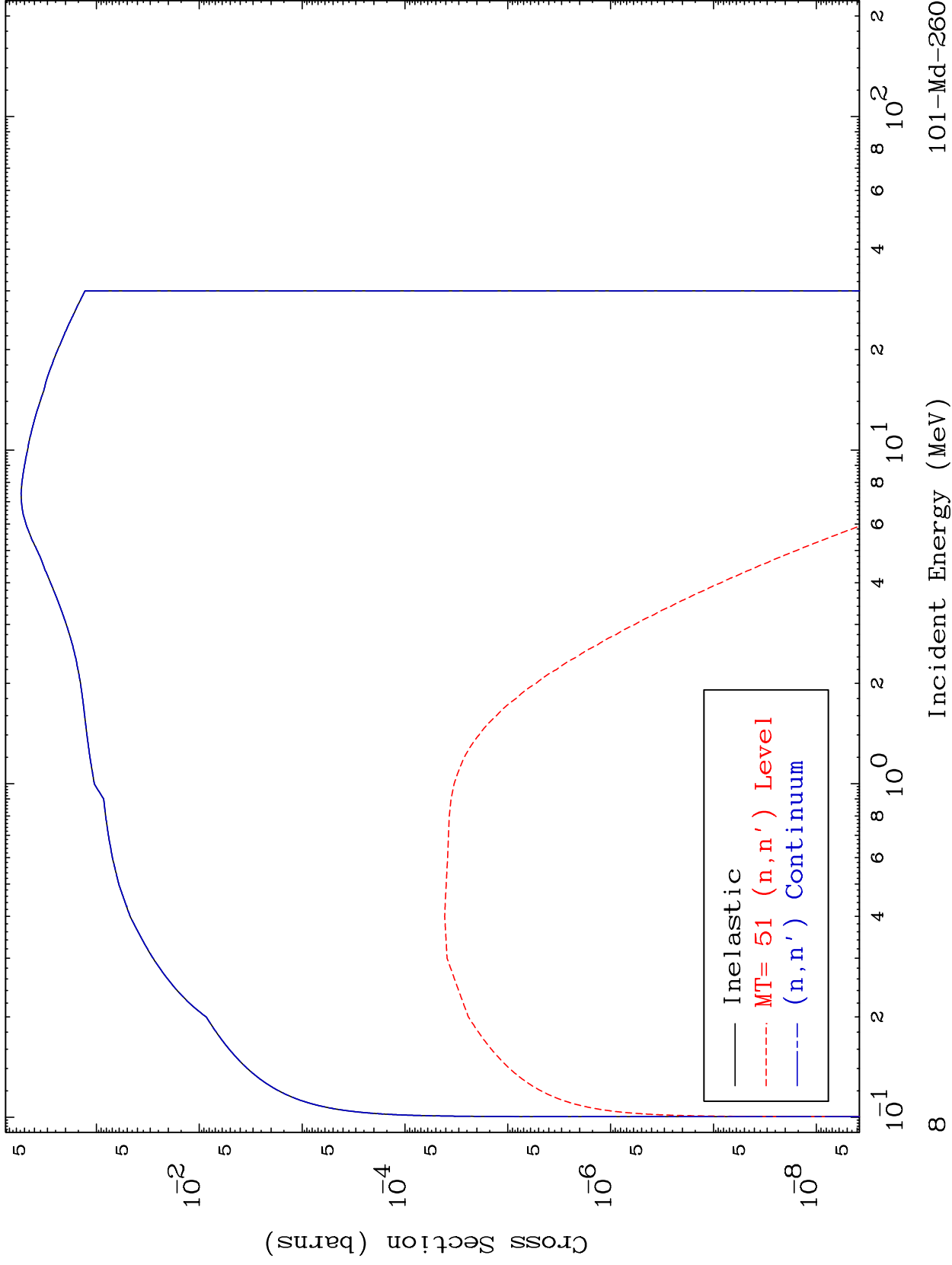
101-Md-260



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(n,n') Level
293 Kelvin Cross Sections

101-Md-260



101-Md-260

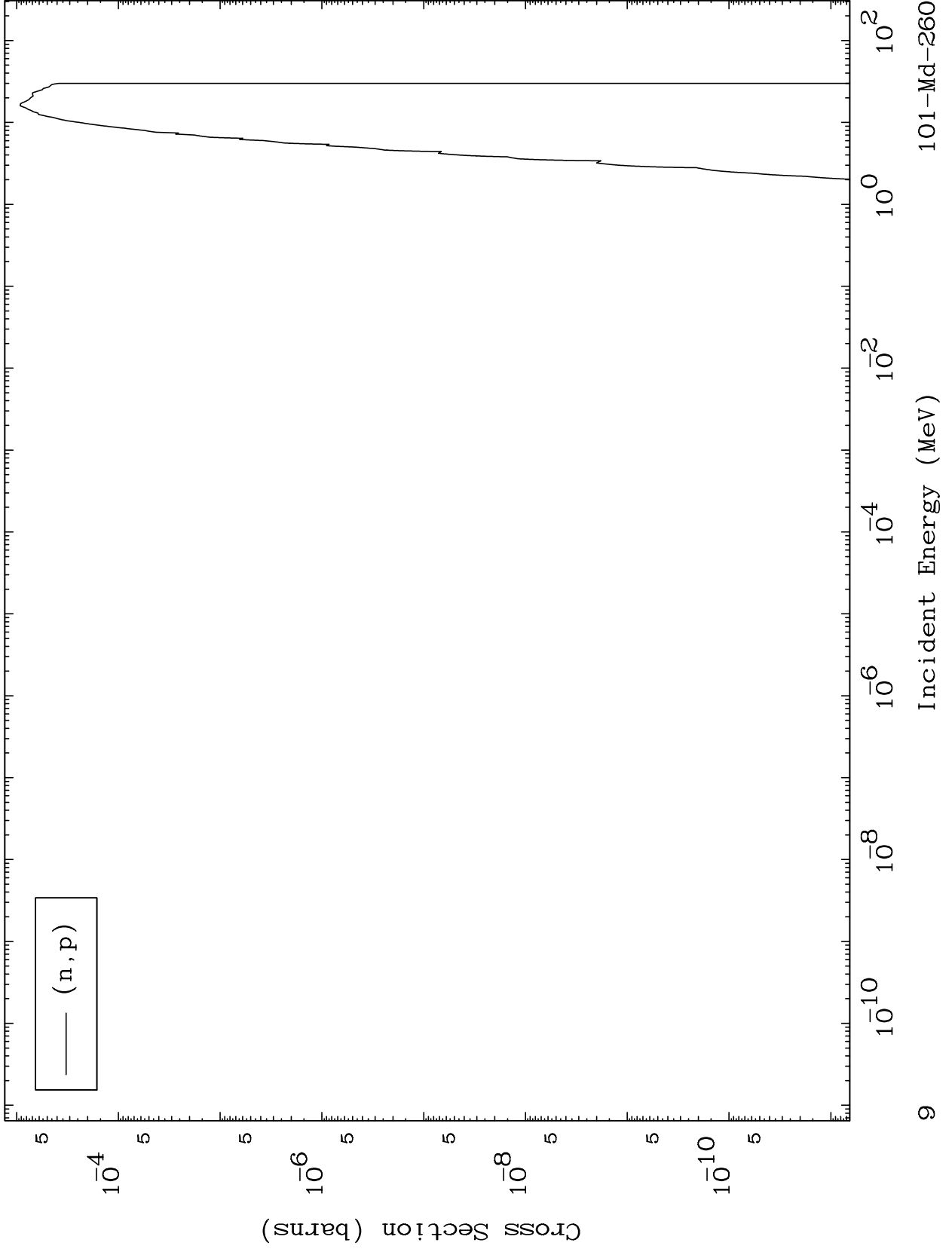
Incident Energy (MeV)

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(n,p) Levels
293 Kelvin Cross Sections

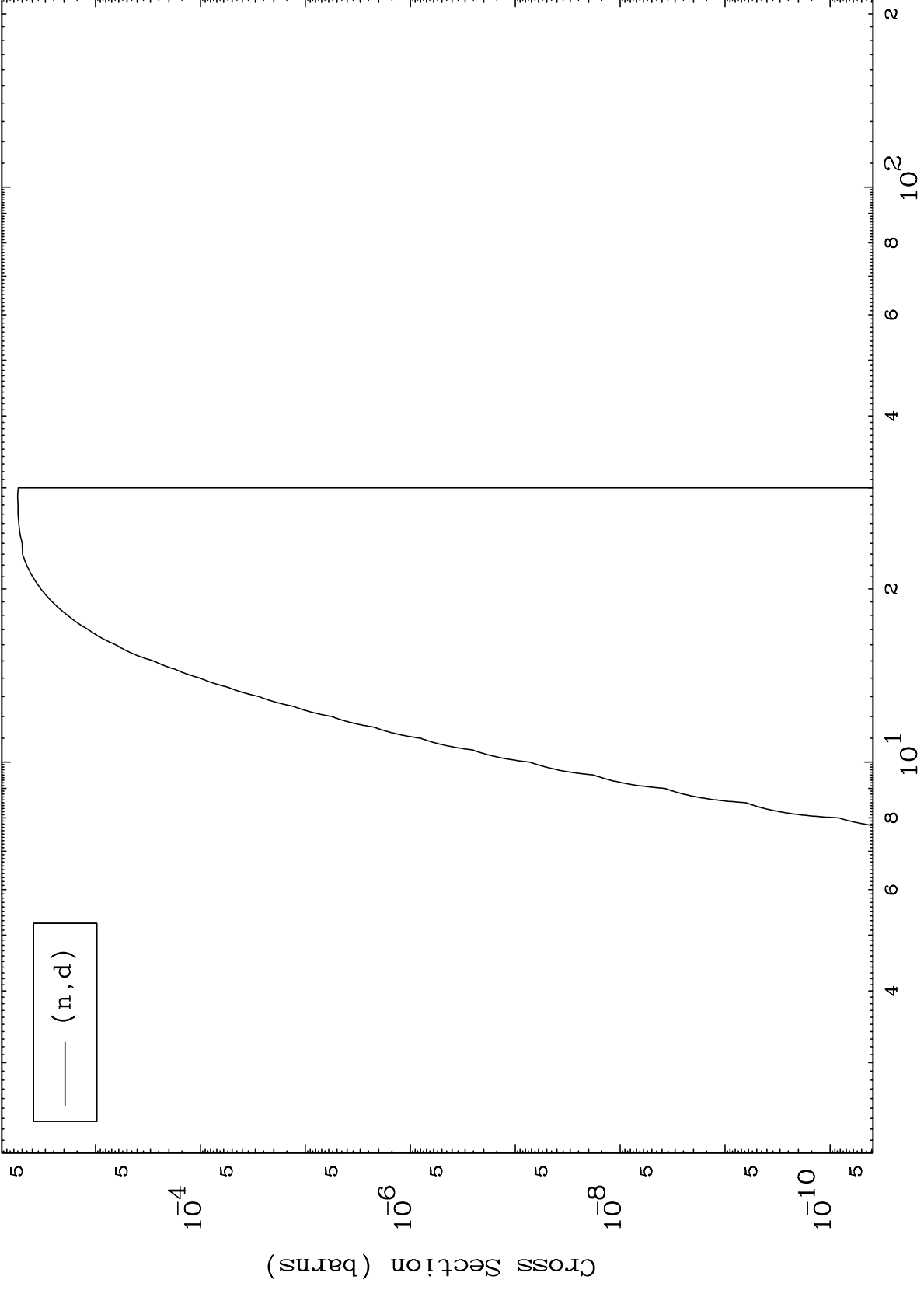
101-Md-260



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-260



10

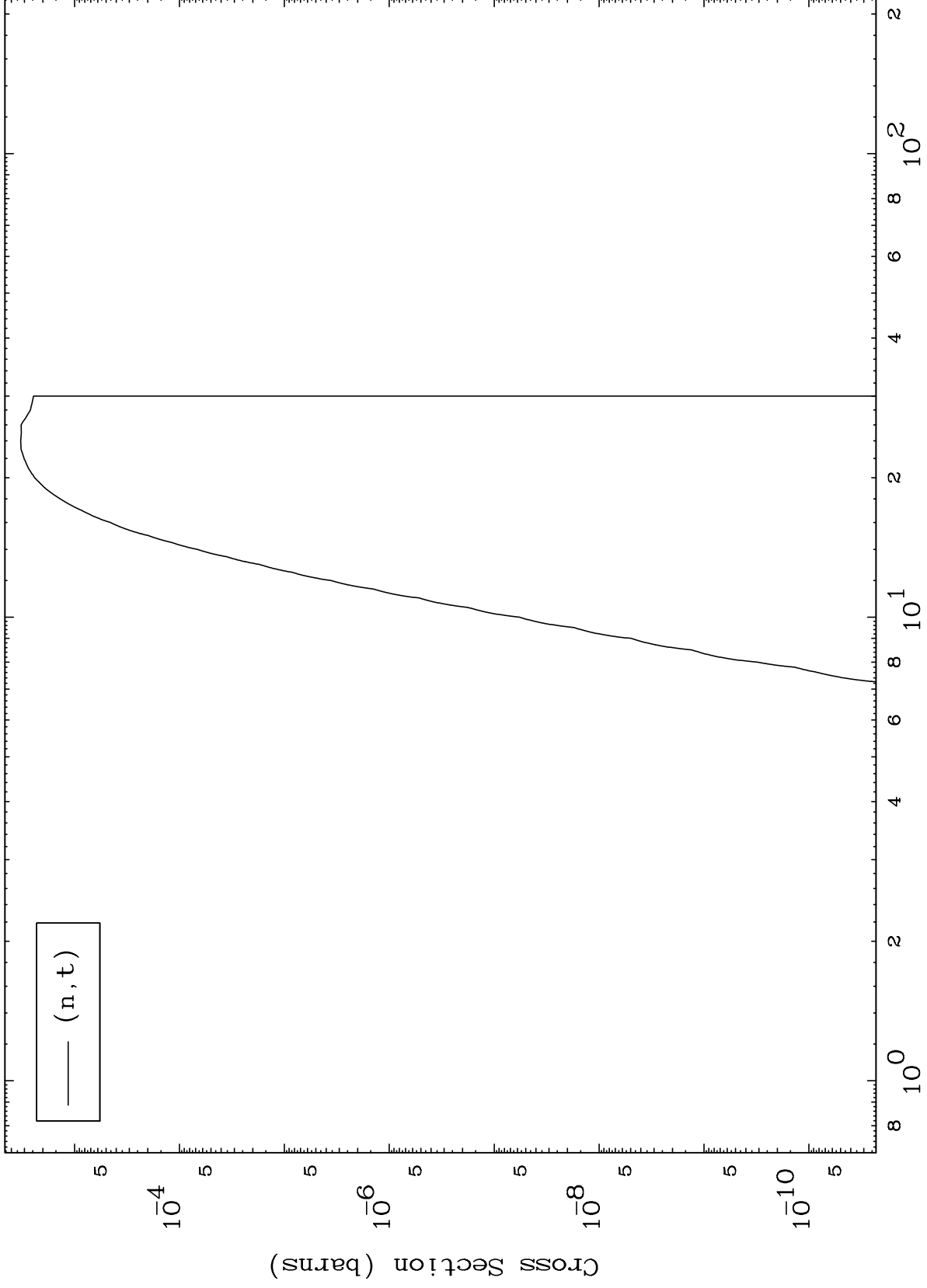
Incident Energy (MeV)

101-Md-260

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(n,t) Levels
293 Kelvin Cross Sections

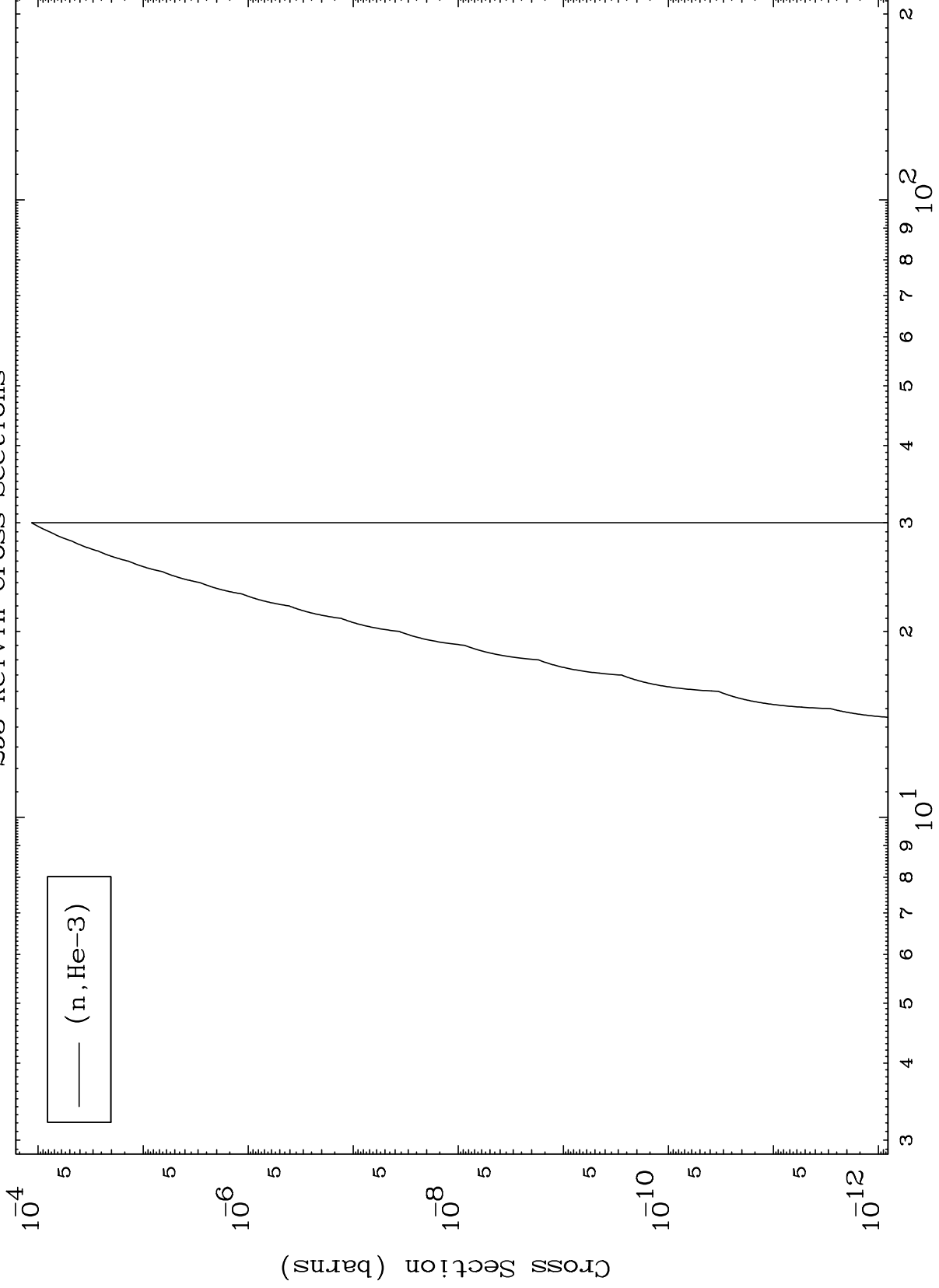
101-Md-260



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(n,He3) Levels
293 Kelvin Cross Sections

101-Md-260



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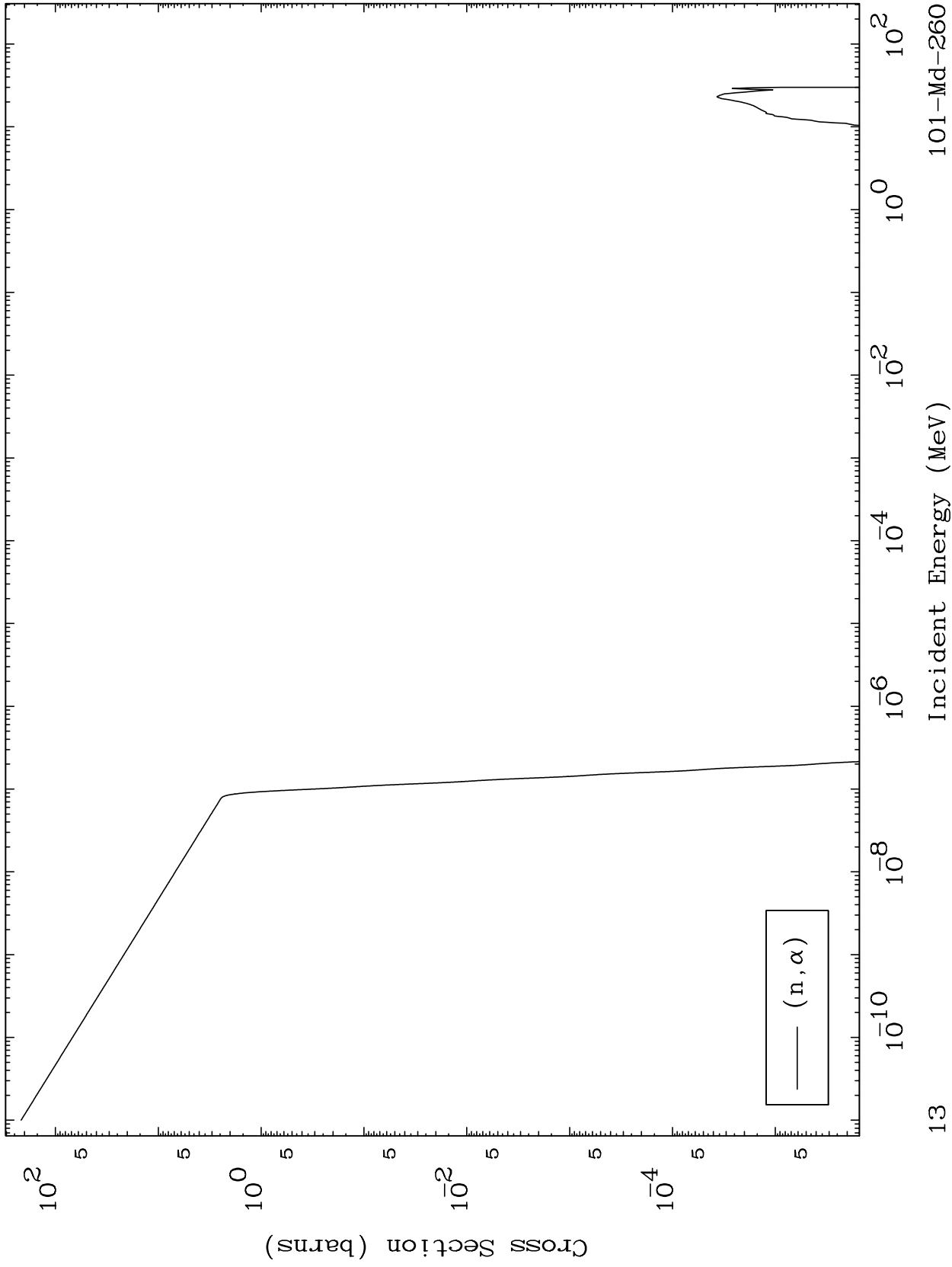
Incident Energy (MeV)

101-Md-260

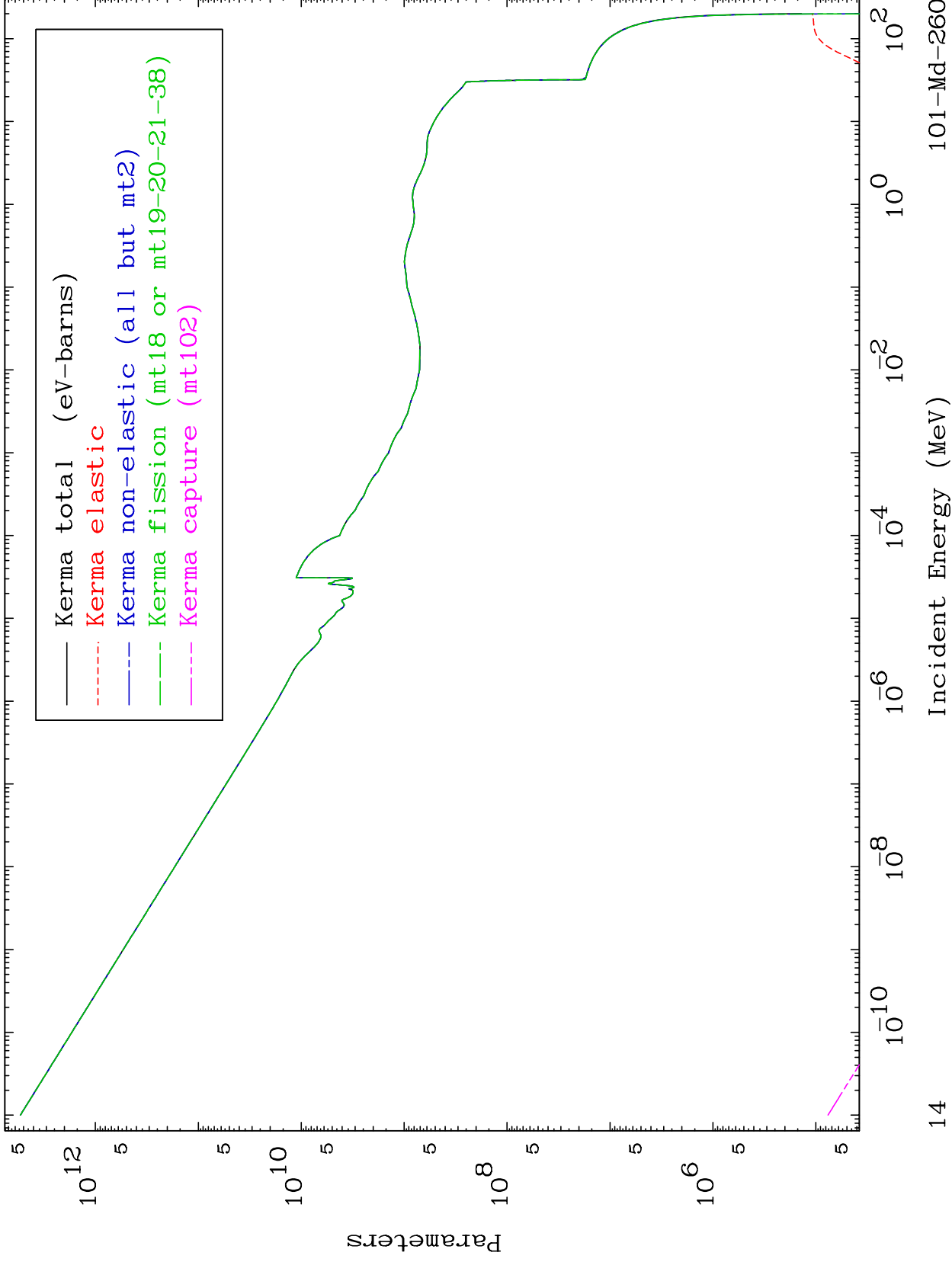
MAT 9961

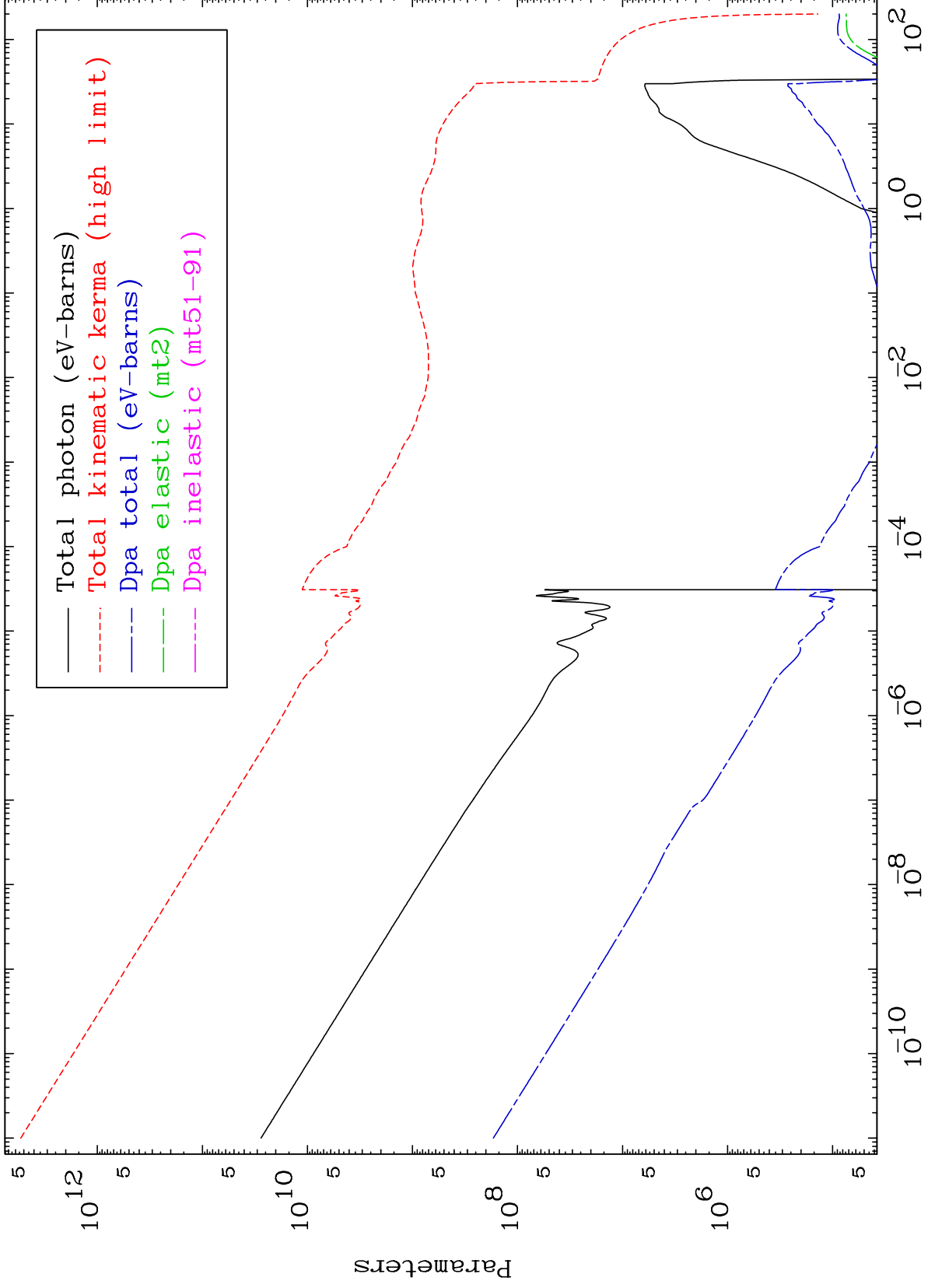
(n, α) Levels
293 Kelvin Cross Sections

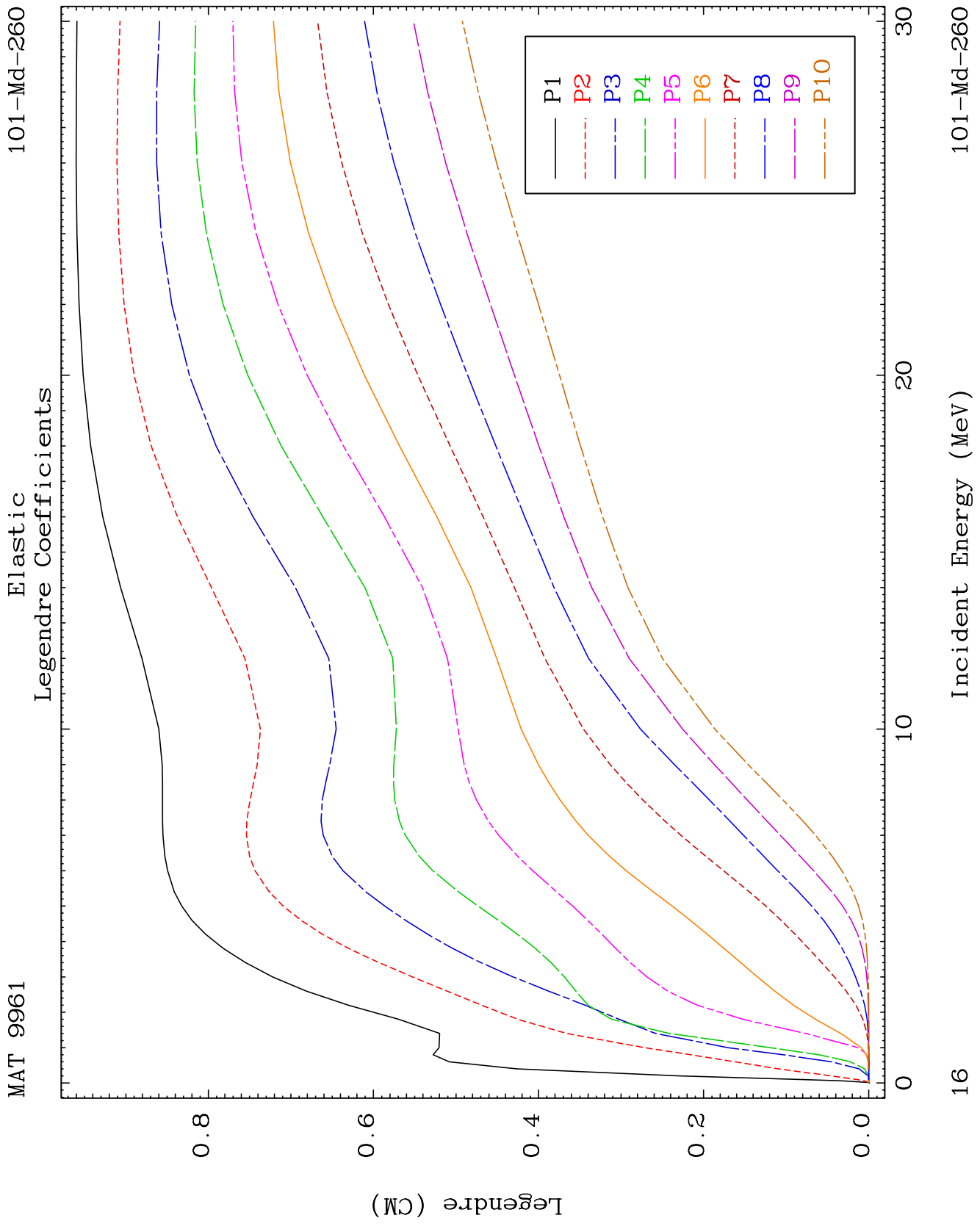
101-Md-260

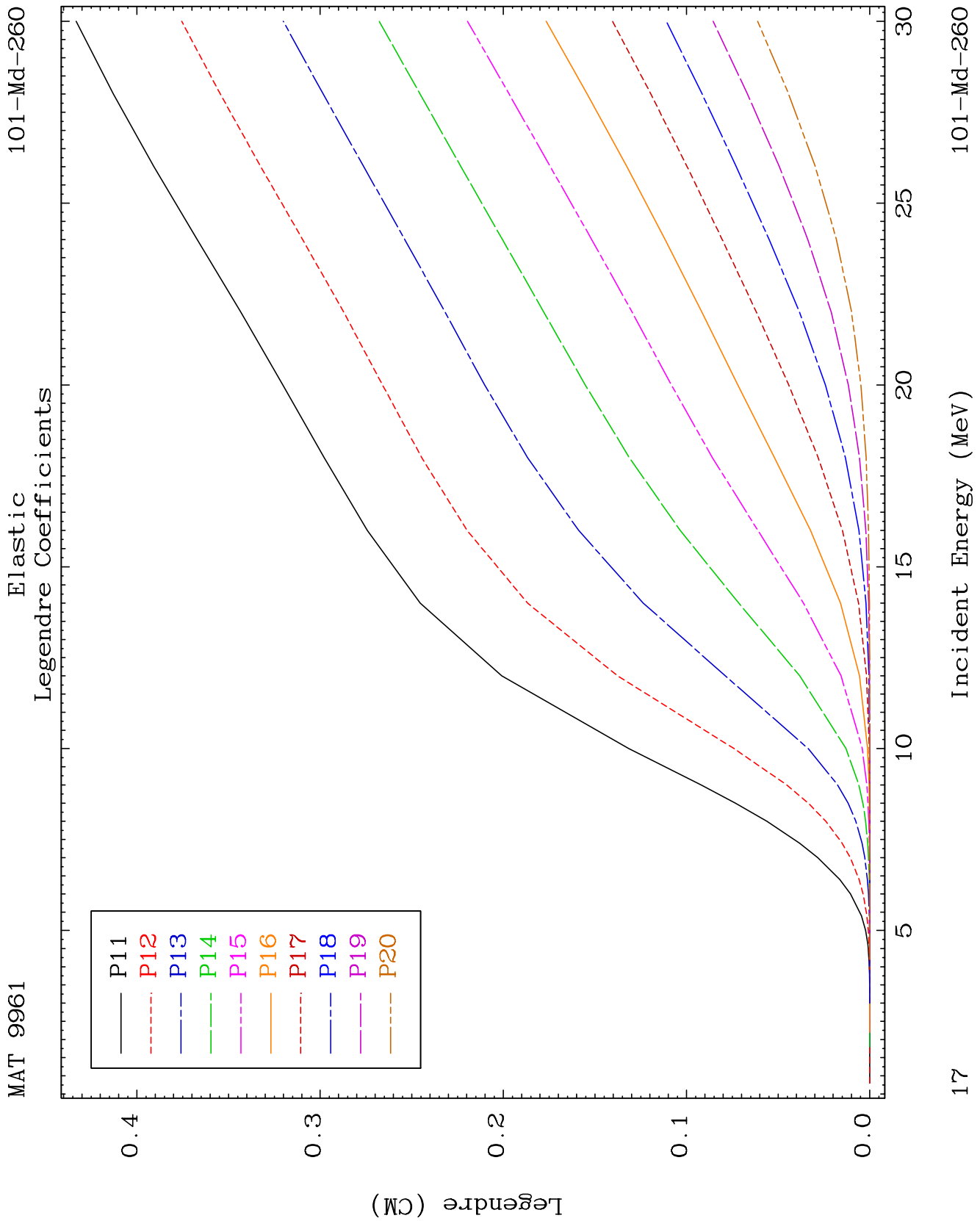


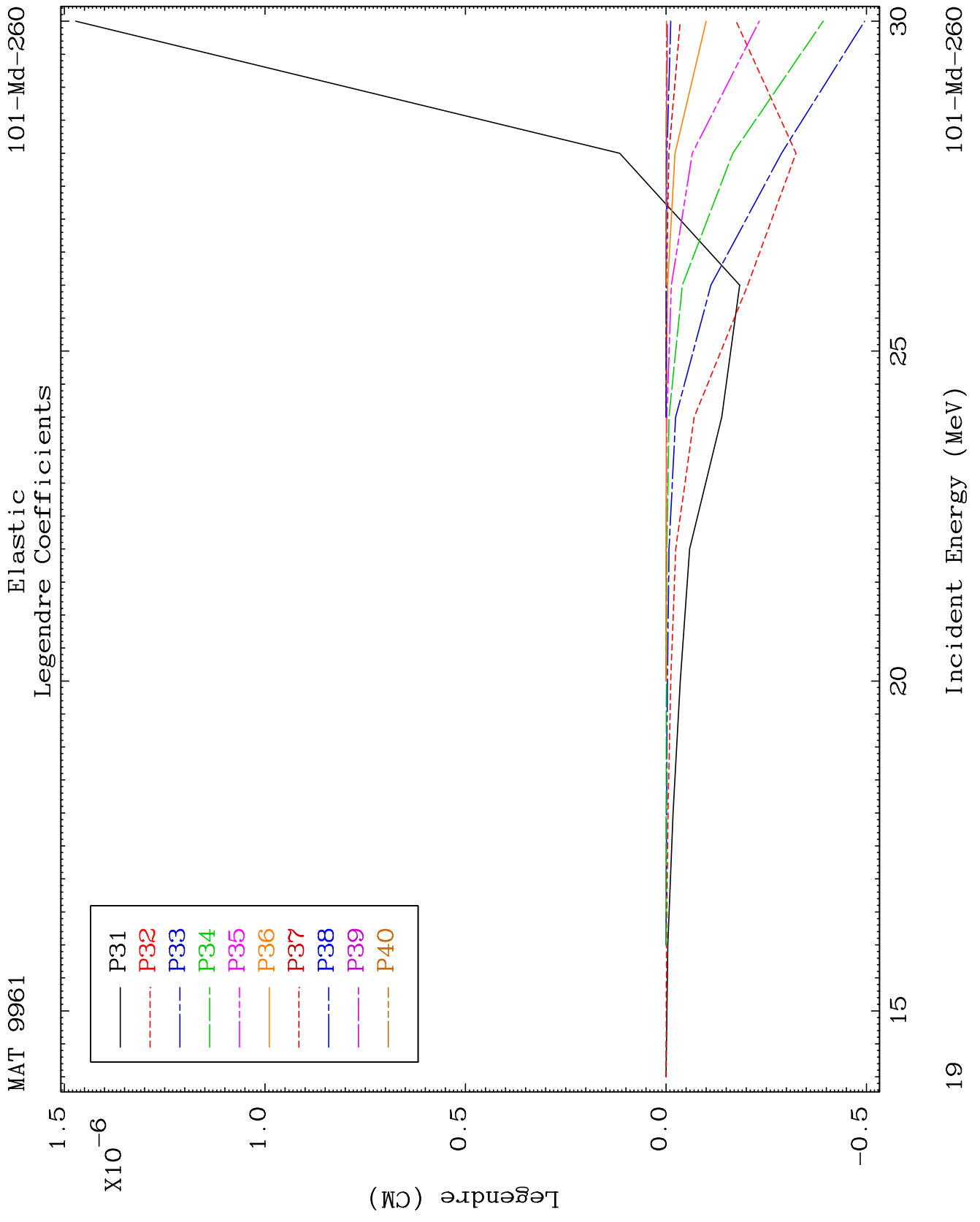
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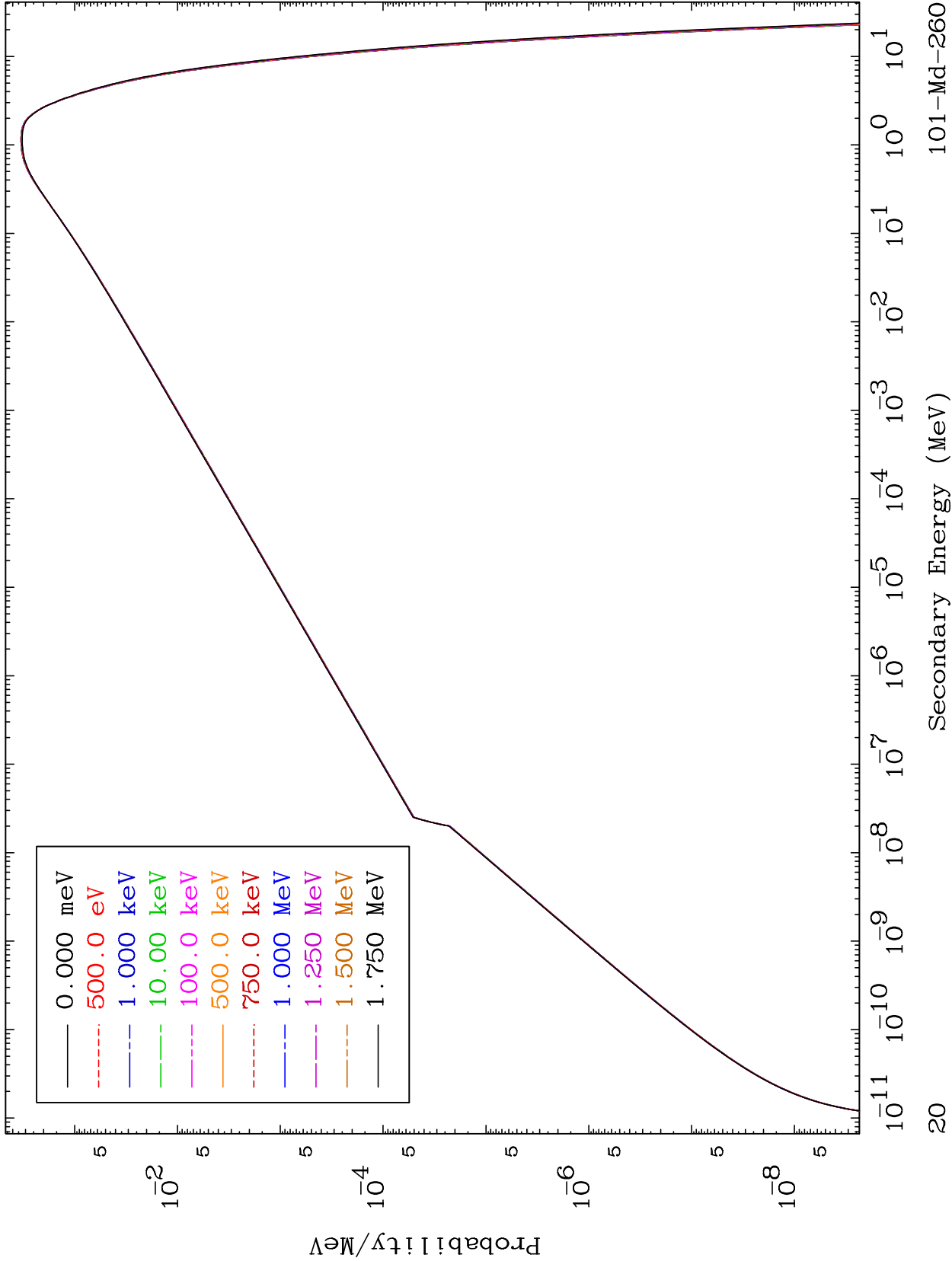




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Fission Energy Distribution

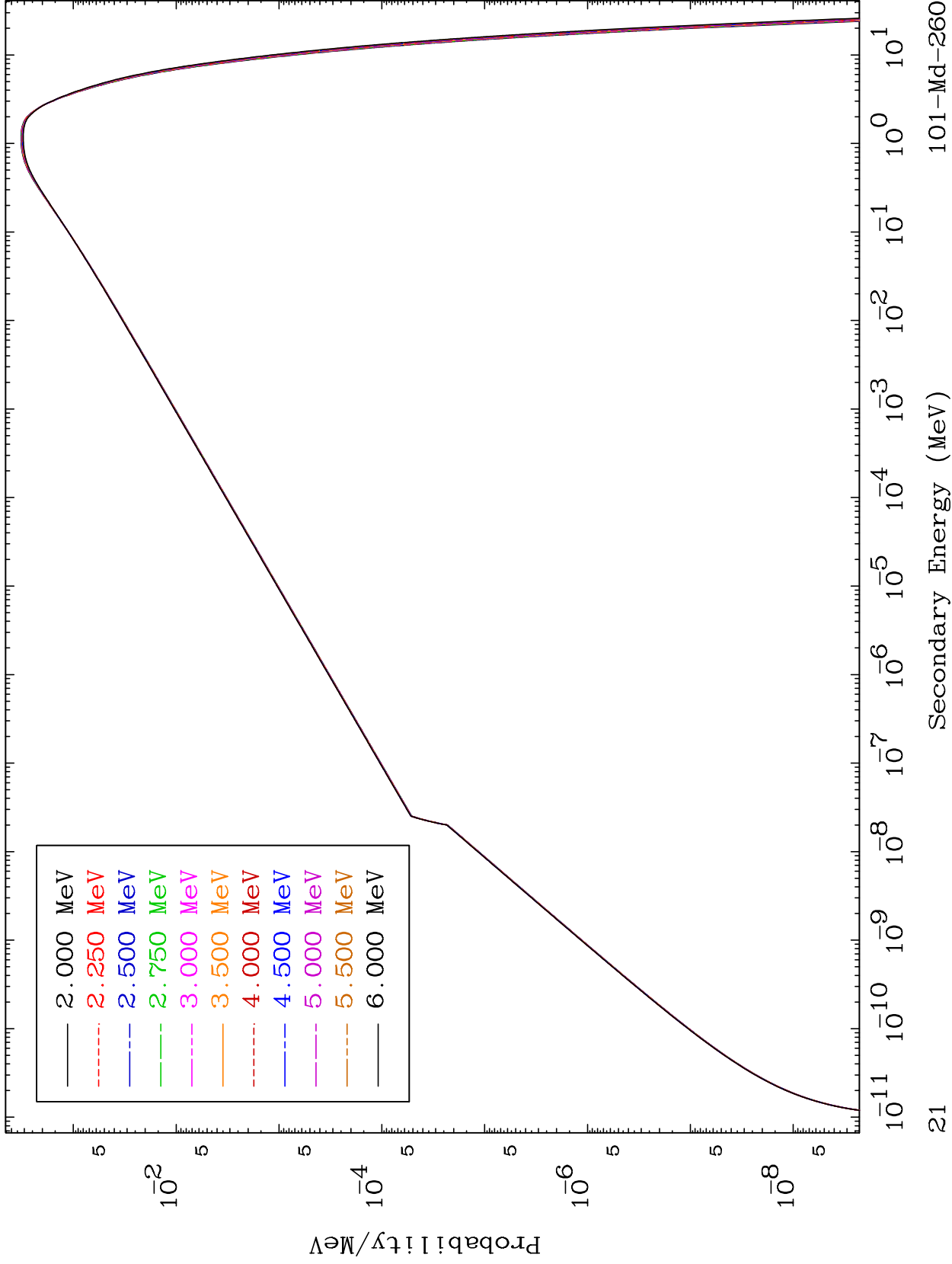
101-Md-260



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Fission Energy Distribution

101-Md-260

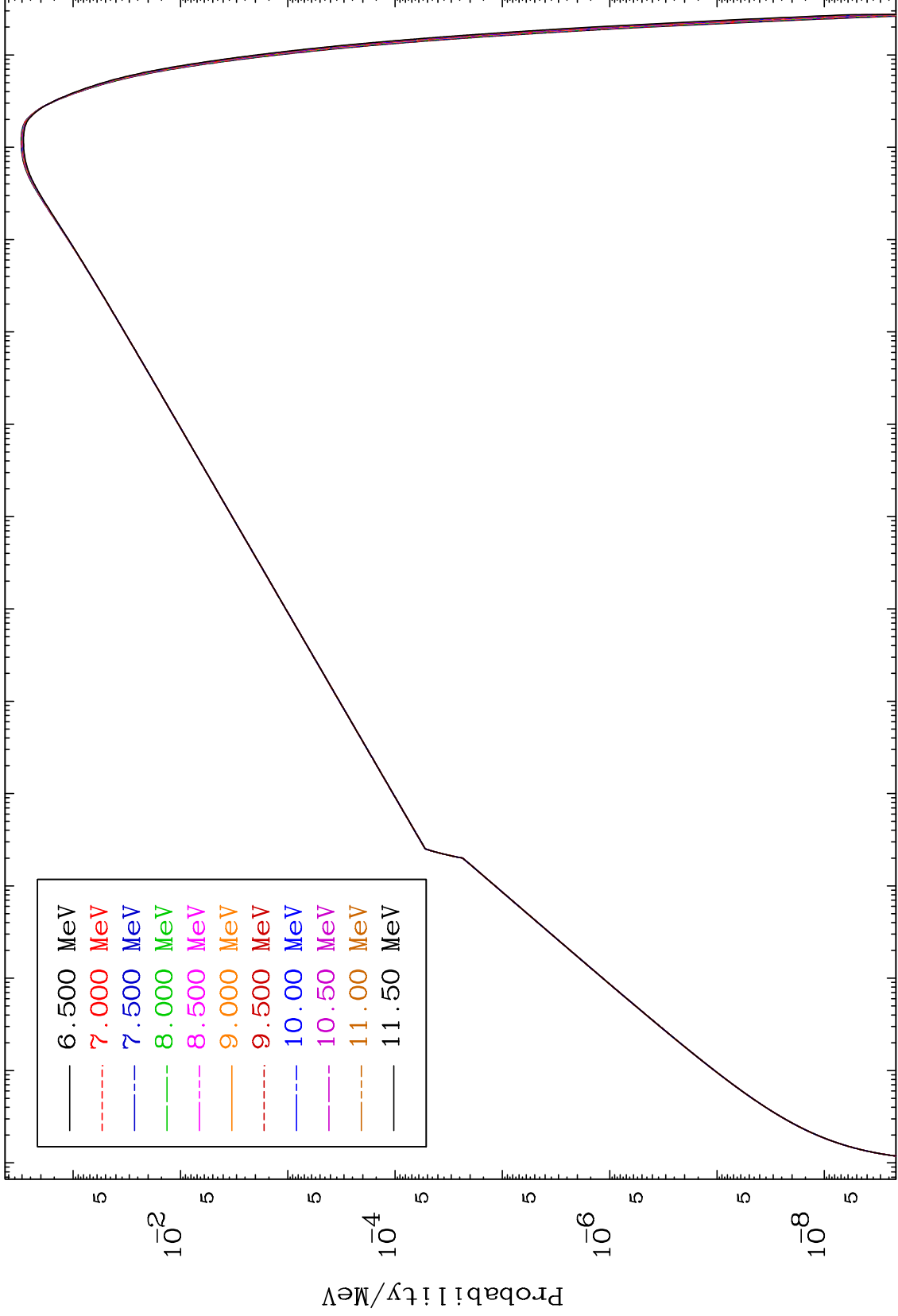


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Fission

101-Md-260

Energy Distribution



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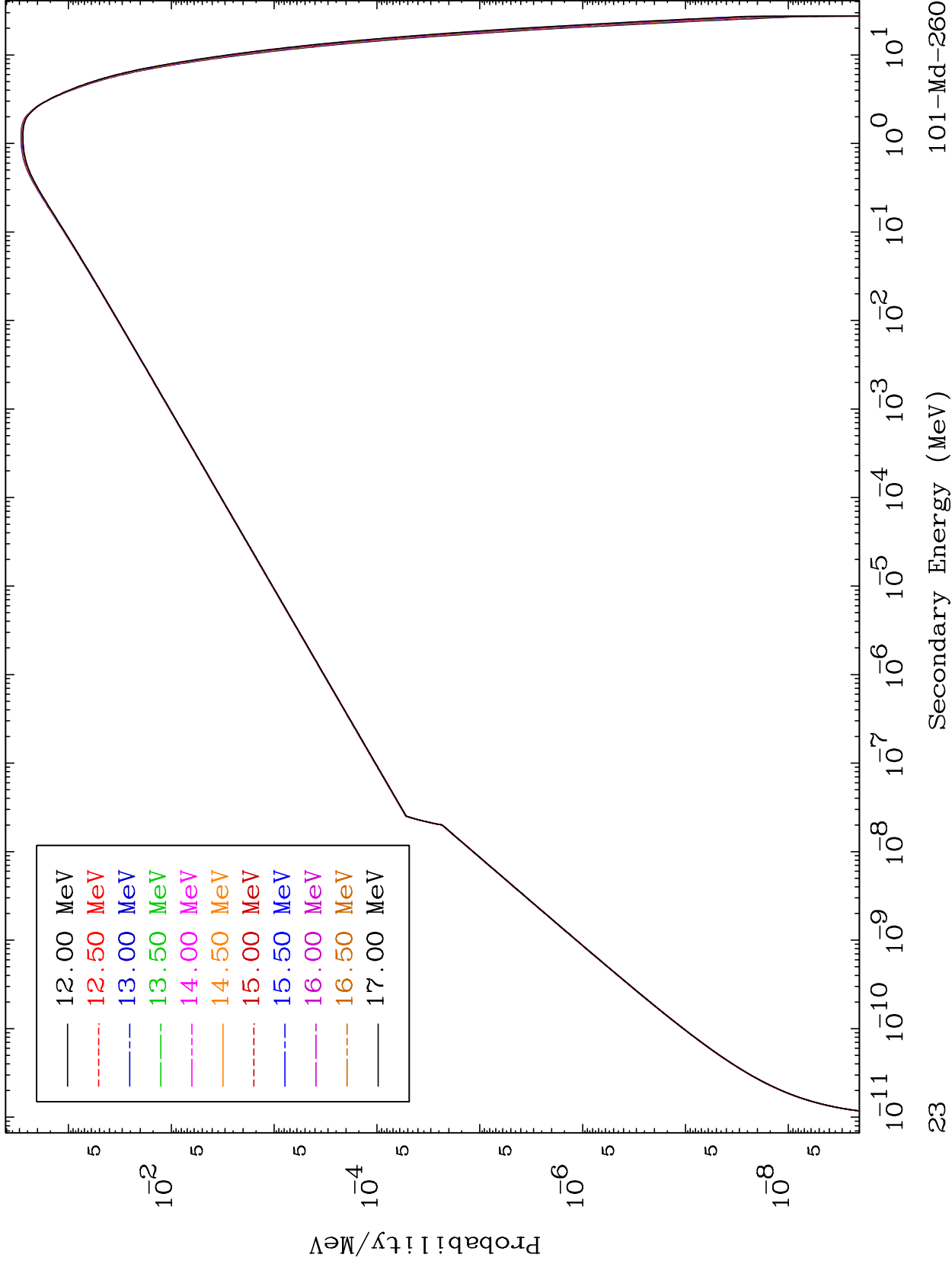
Secondary Energy (MeV)

101-Md-260

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Fission Energy Distribution

101-Md-260



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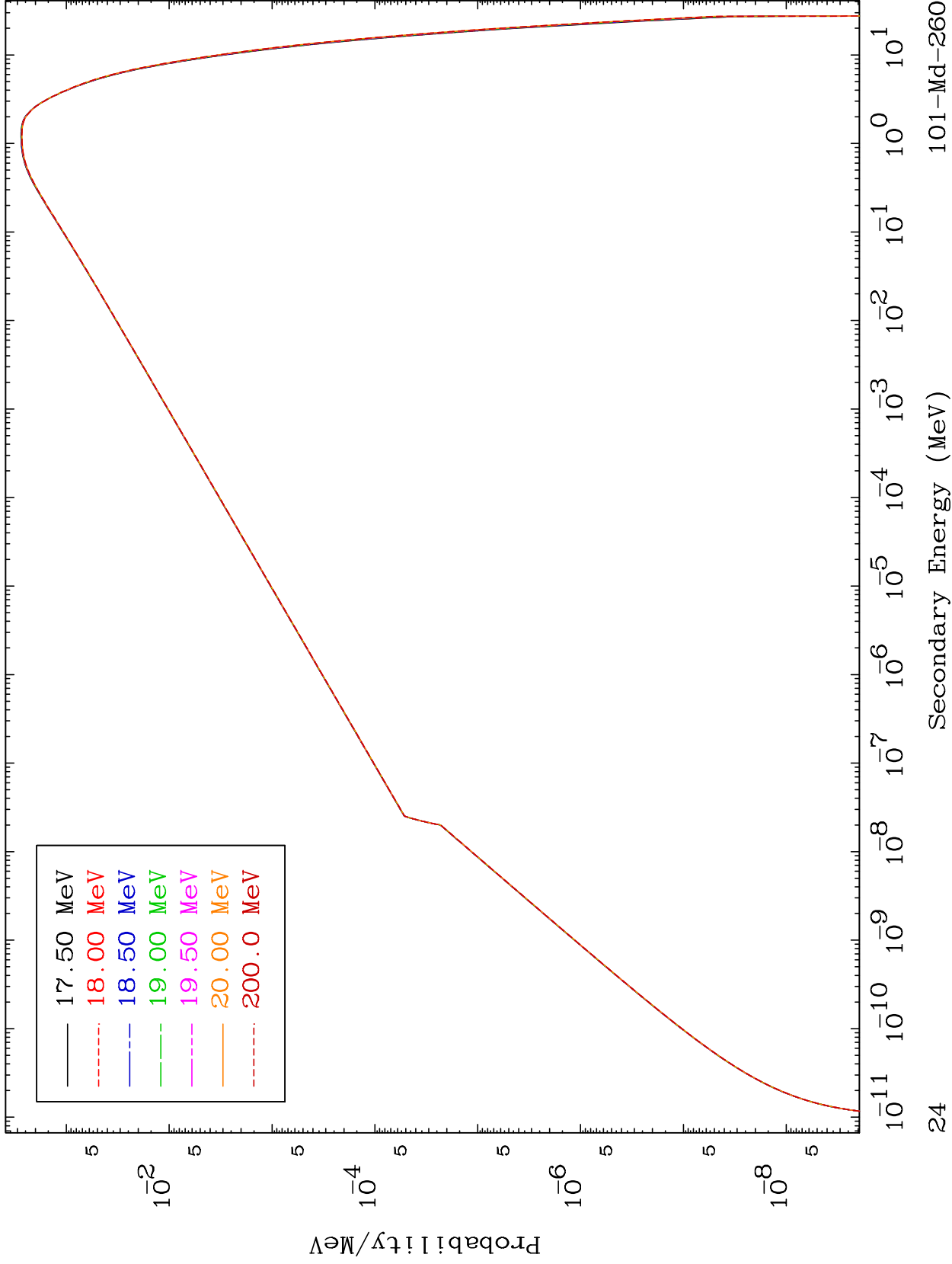
Secondary Energy (MeV)

101-Md-260

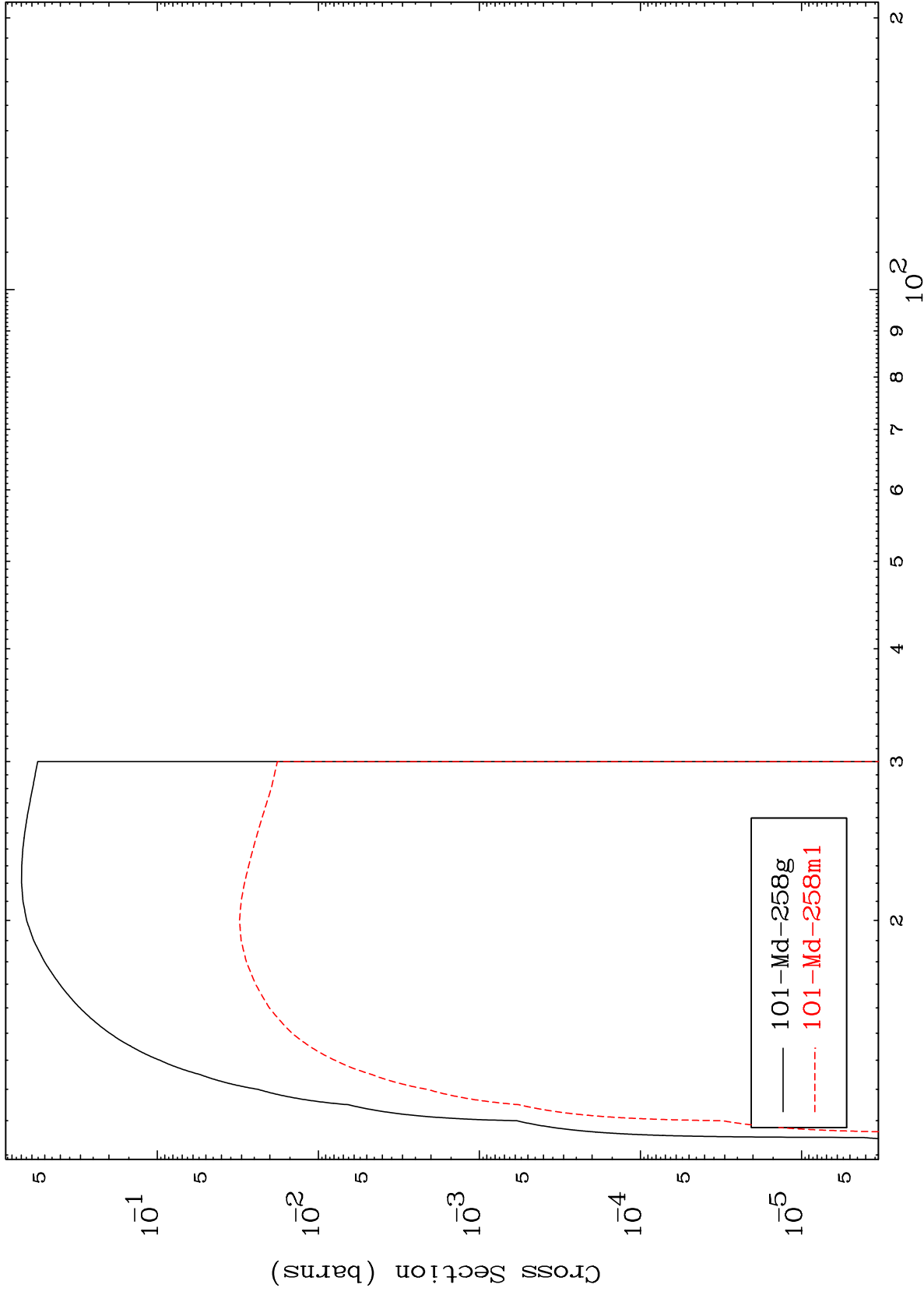
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Fission Energy Distribution

101-Md-260



(n,3n)
Radionuclide Production Cross Section

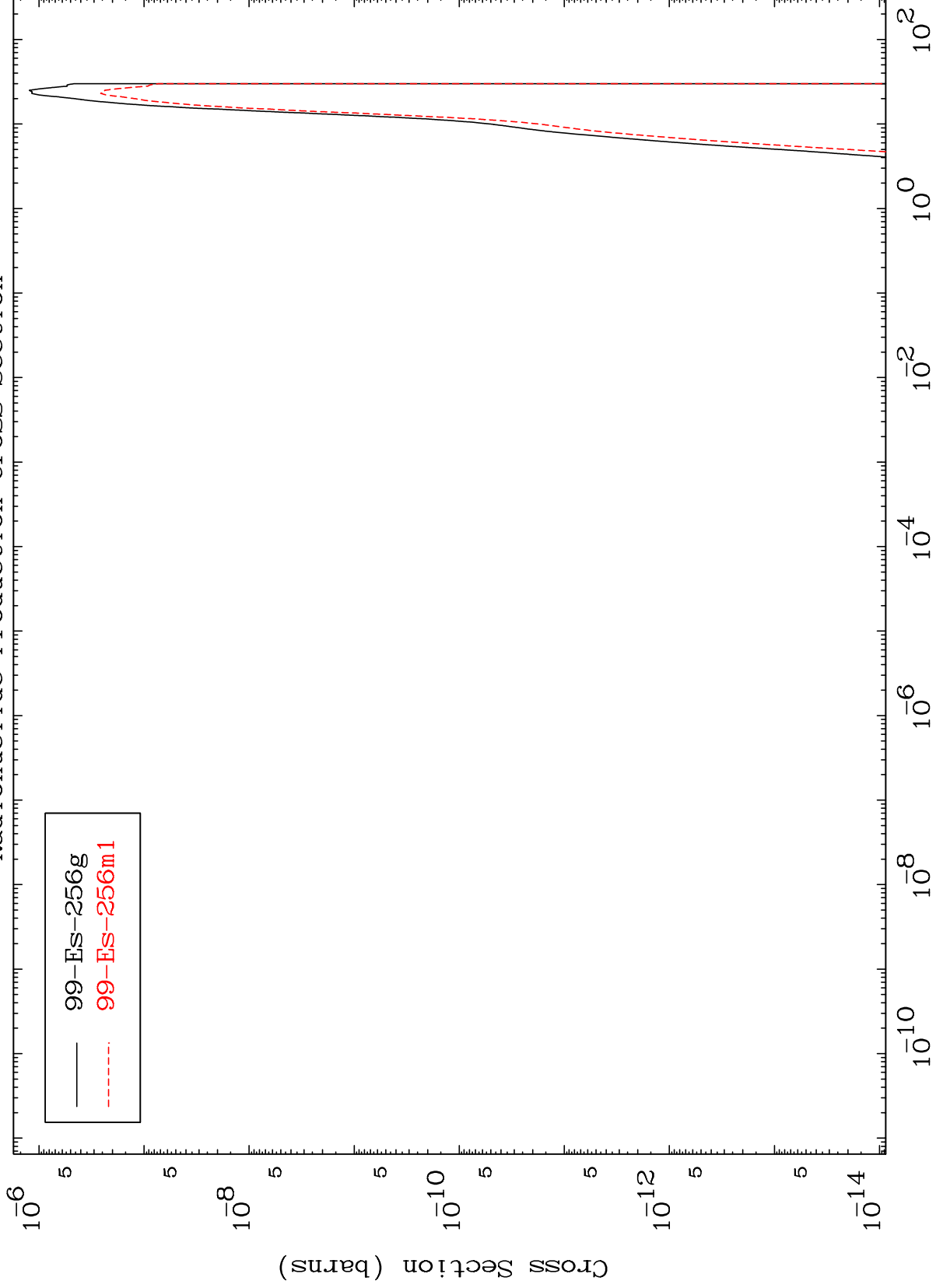


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(n,n') α

101-Md-260

Radionuclide Production Cross Section



99-Es-256g
99-Es-256m1

Incident Energy (MeV)

101-Md-260

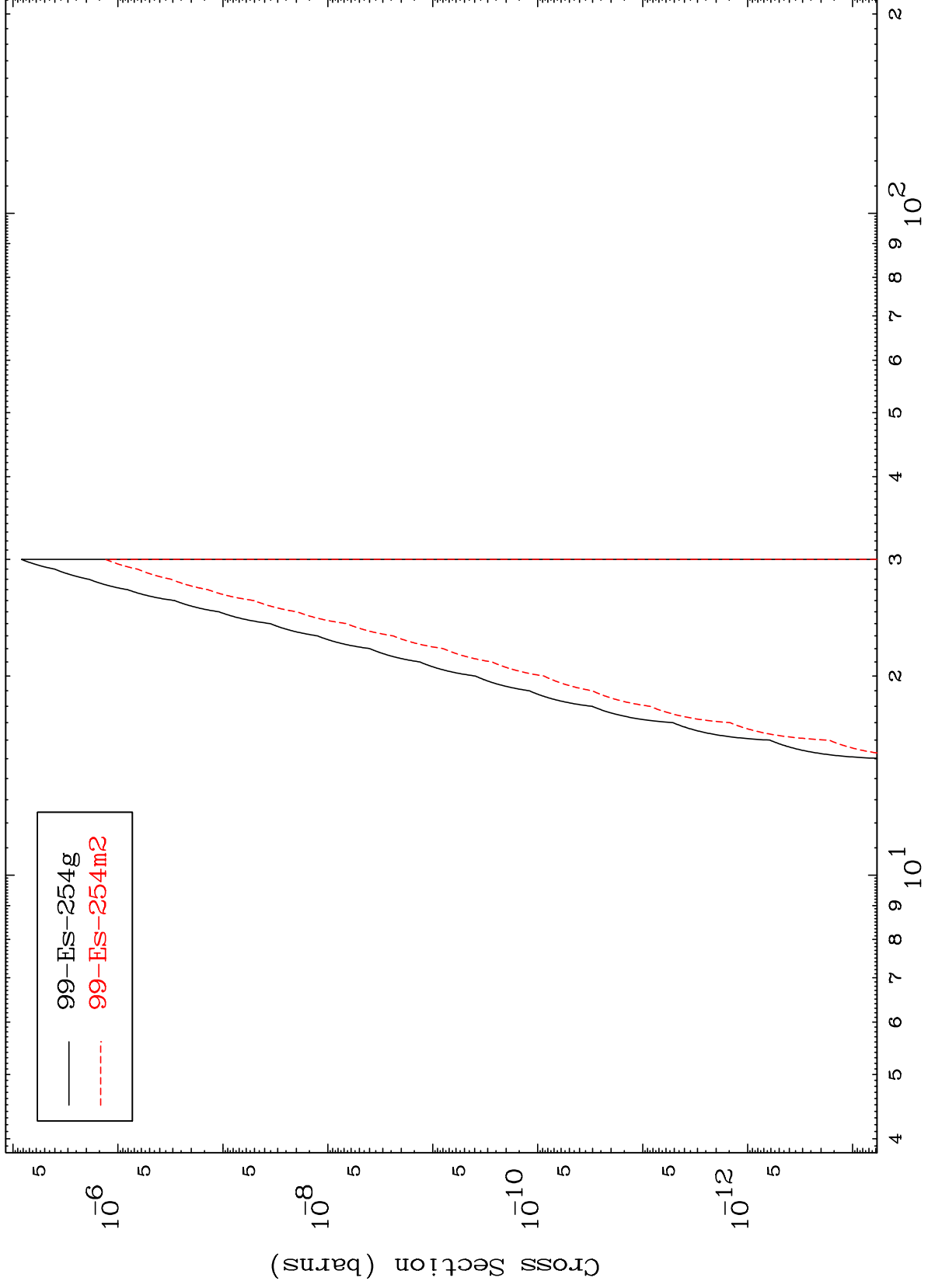
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(n,3n) α

101-Md-260

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-260