

Program EVALPLOT  
(Version 2018-1)

by

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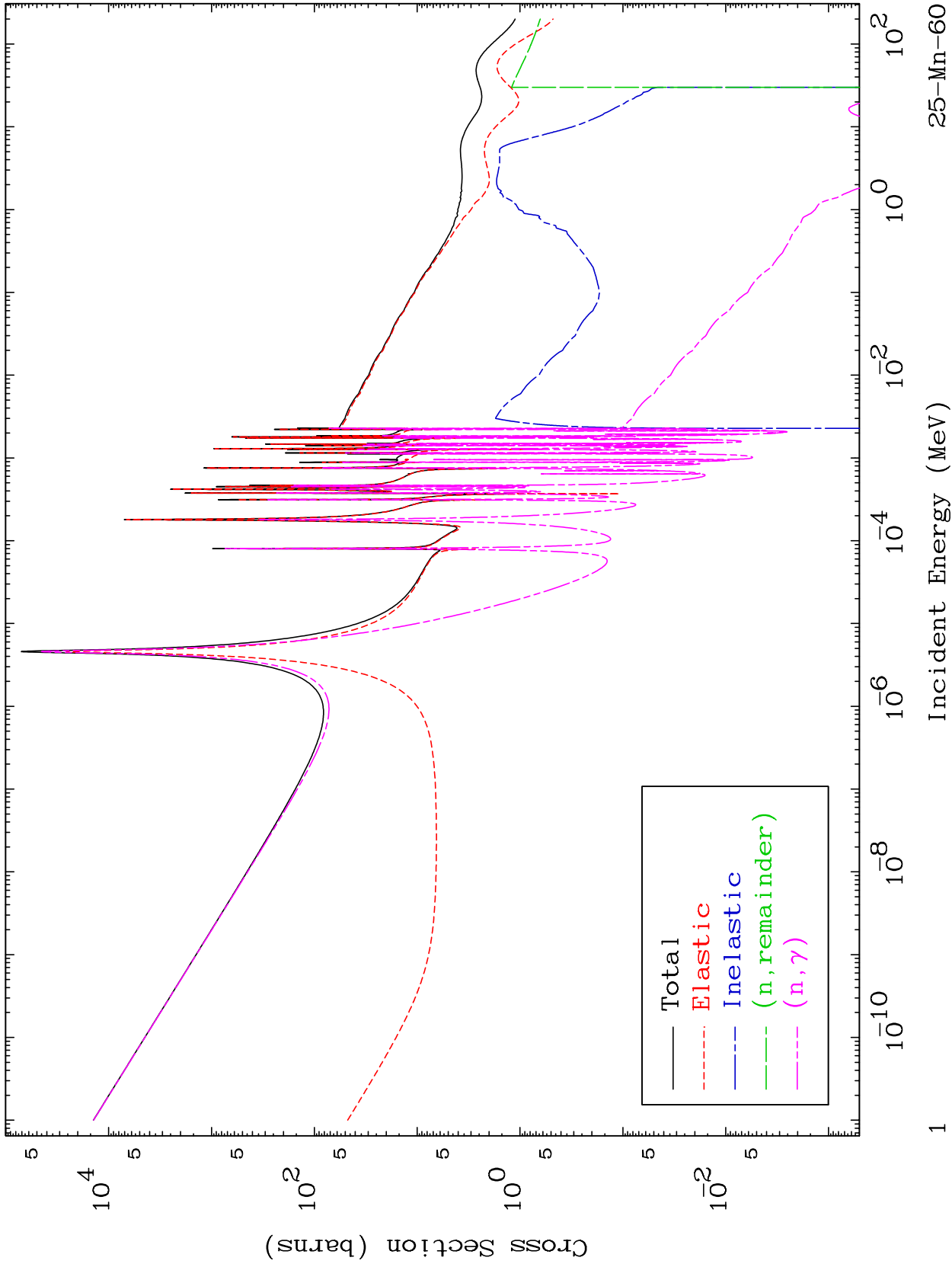
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 2541

Major  
293 Kelvin Cross Sections

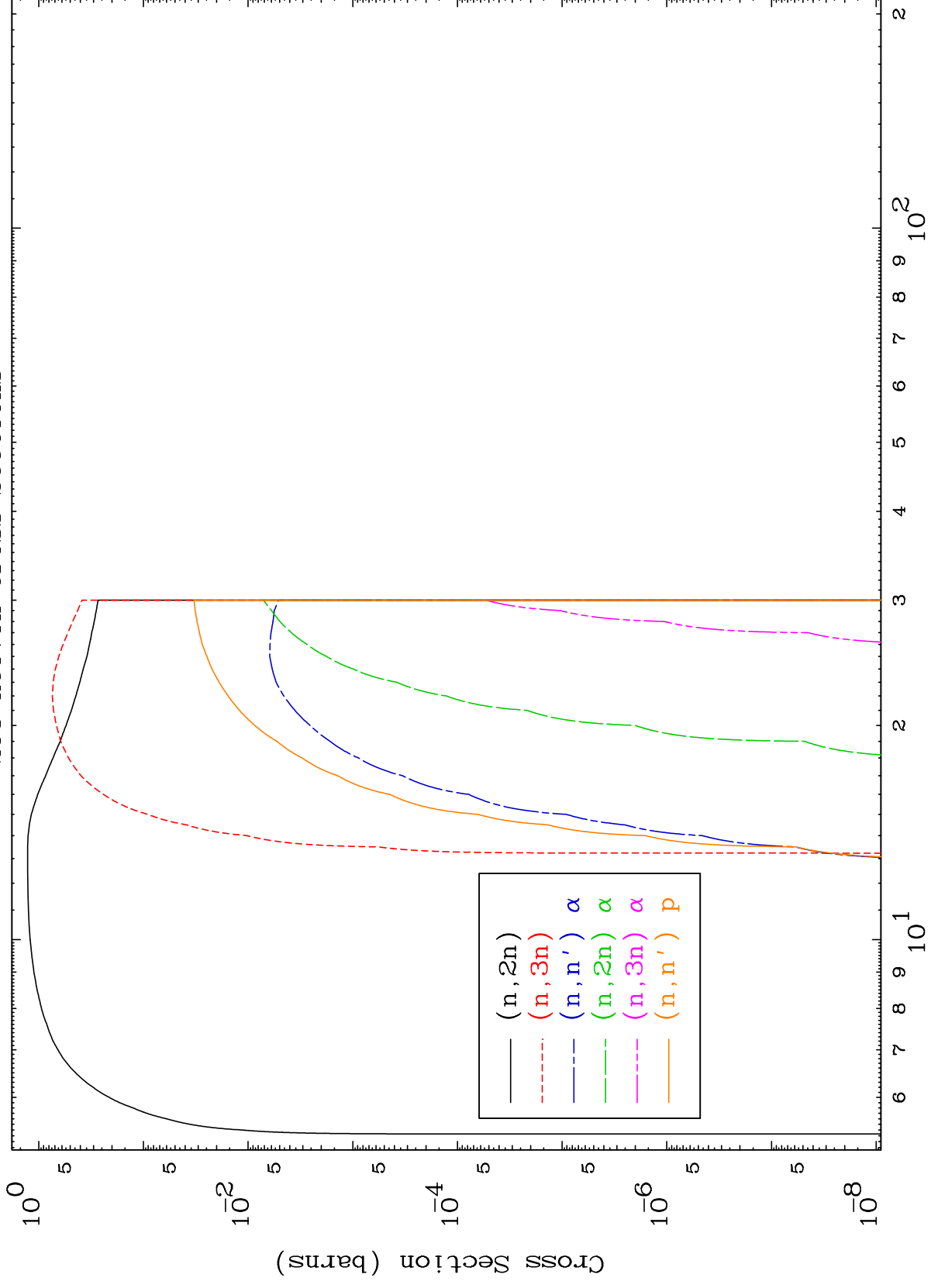
25-Mn-60



MAT 2541

Neutron Production  
293 Kelvin Cross Sections

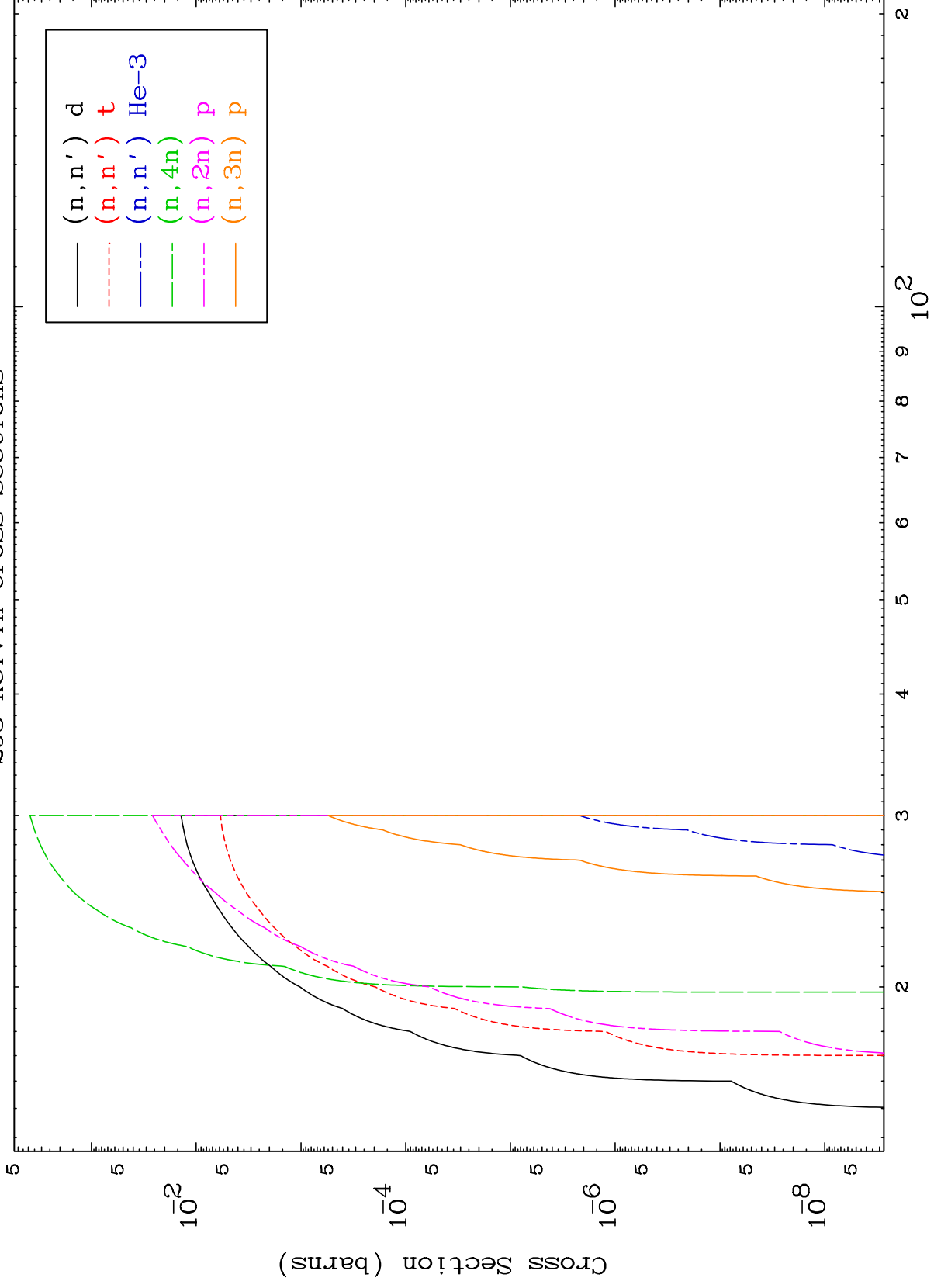
25-Mn-60



MAT 2541

Neutron Production  
293 Kelvin Cross Sections

25-Mn-60



3

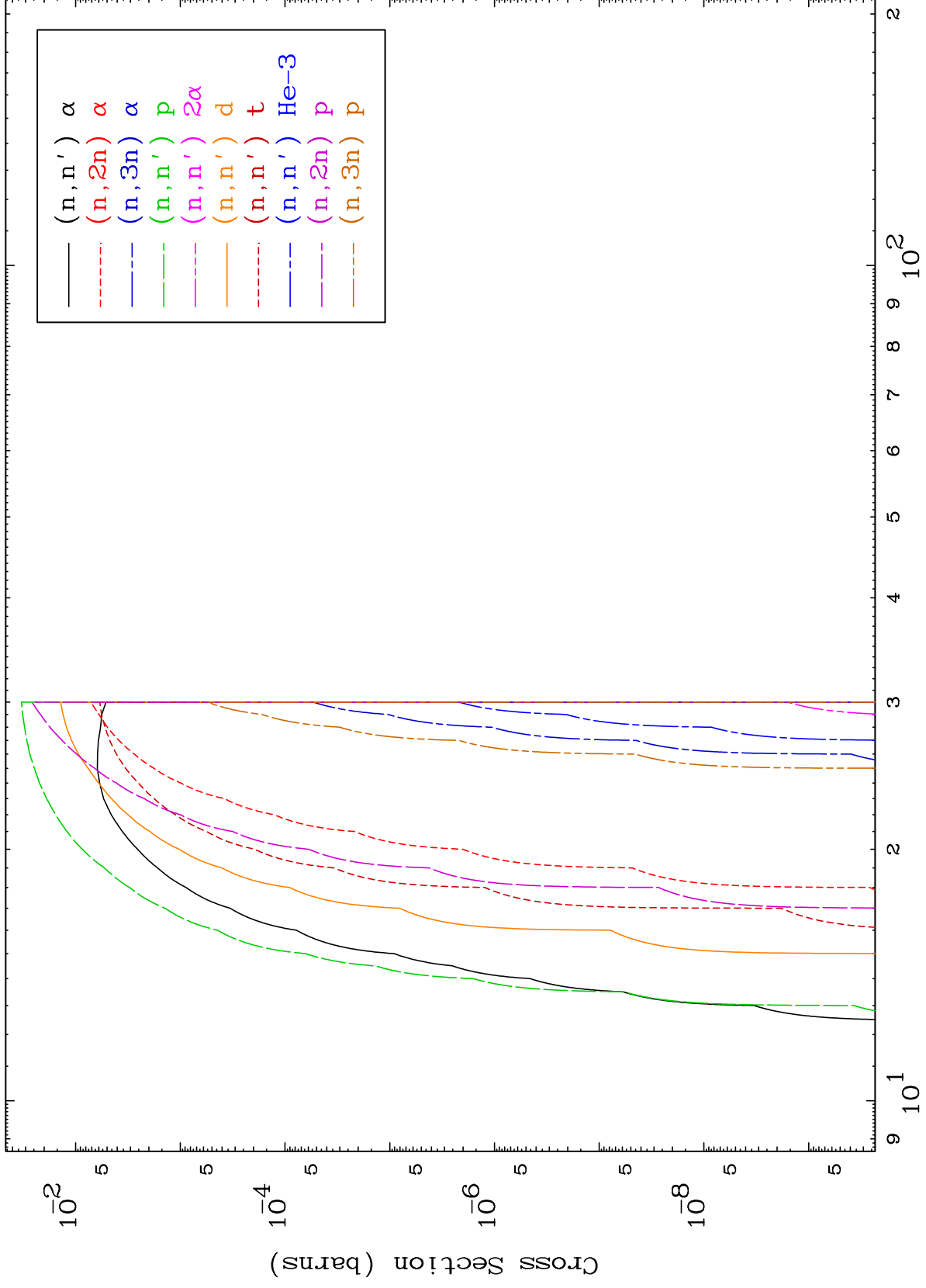
Incident Energy (MeV)

25-Mn-60

MAT 2541

Charged Particle  
293 Kelvin Cross Sections

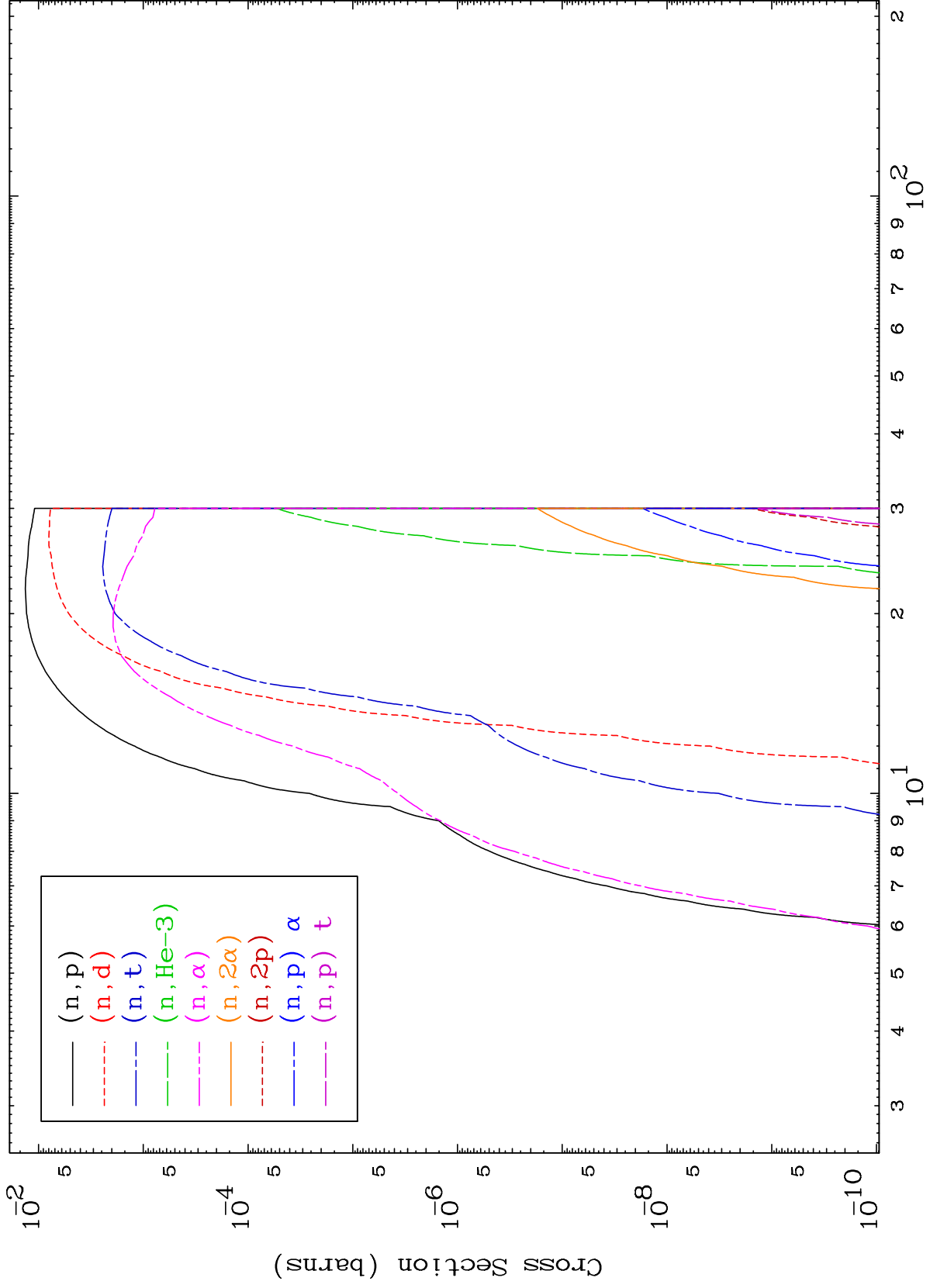
25-Mn-60



Incident Energy (MeV)

25-Mn-60

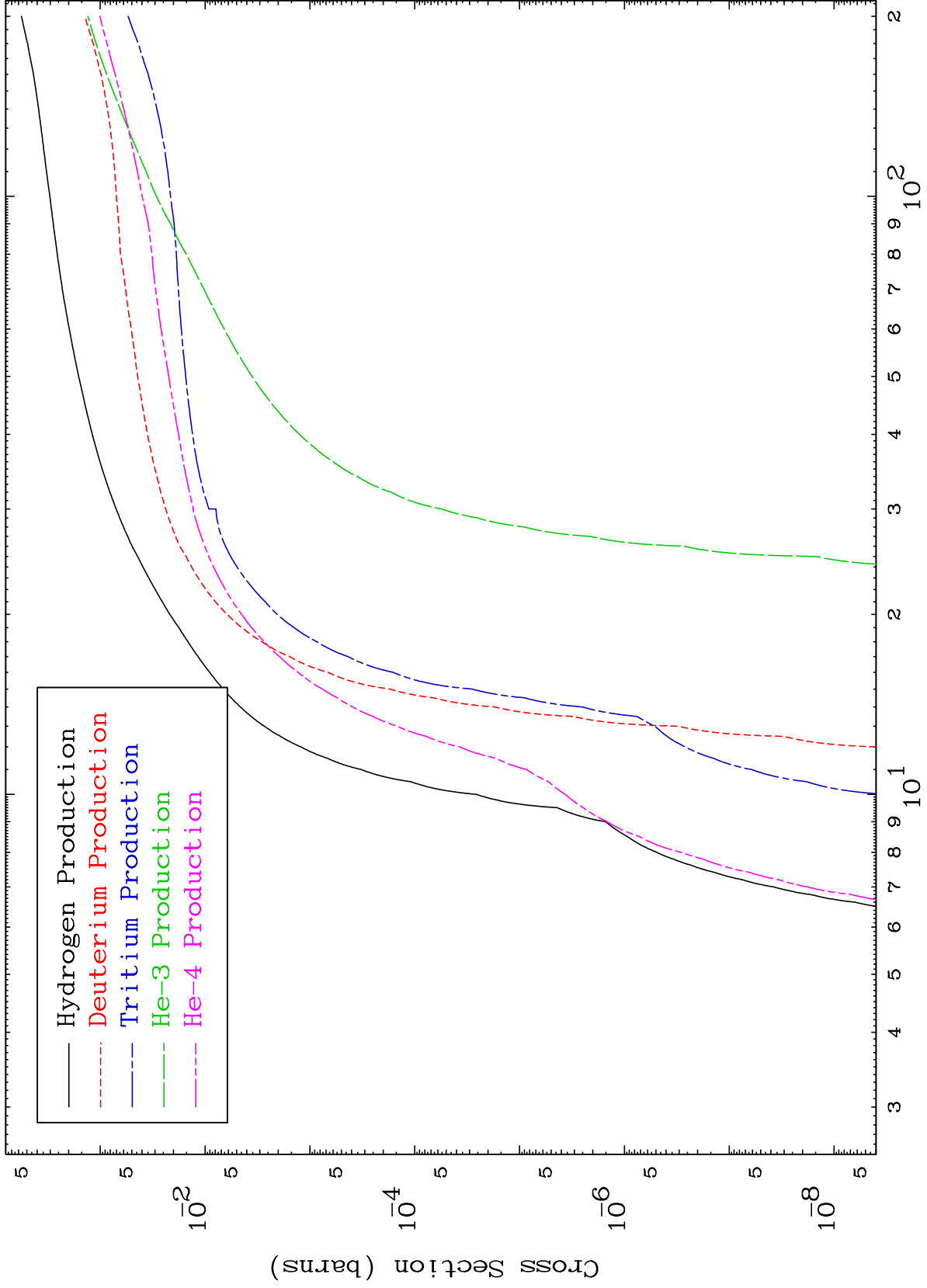
4



MAT 2541

Particle Production  
293 Kelvin Cross Sections

25-Mn-60



6

Incident Energy (MeV)

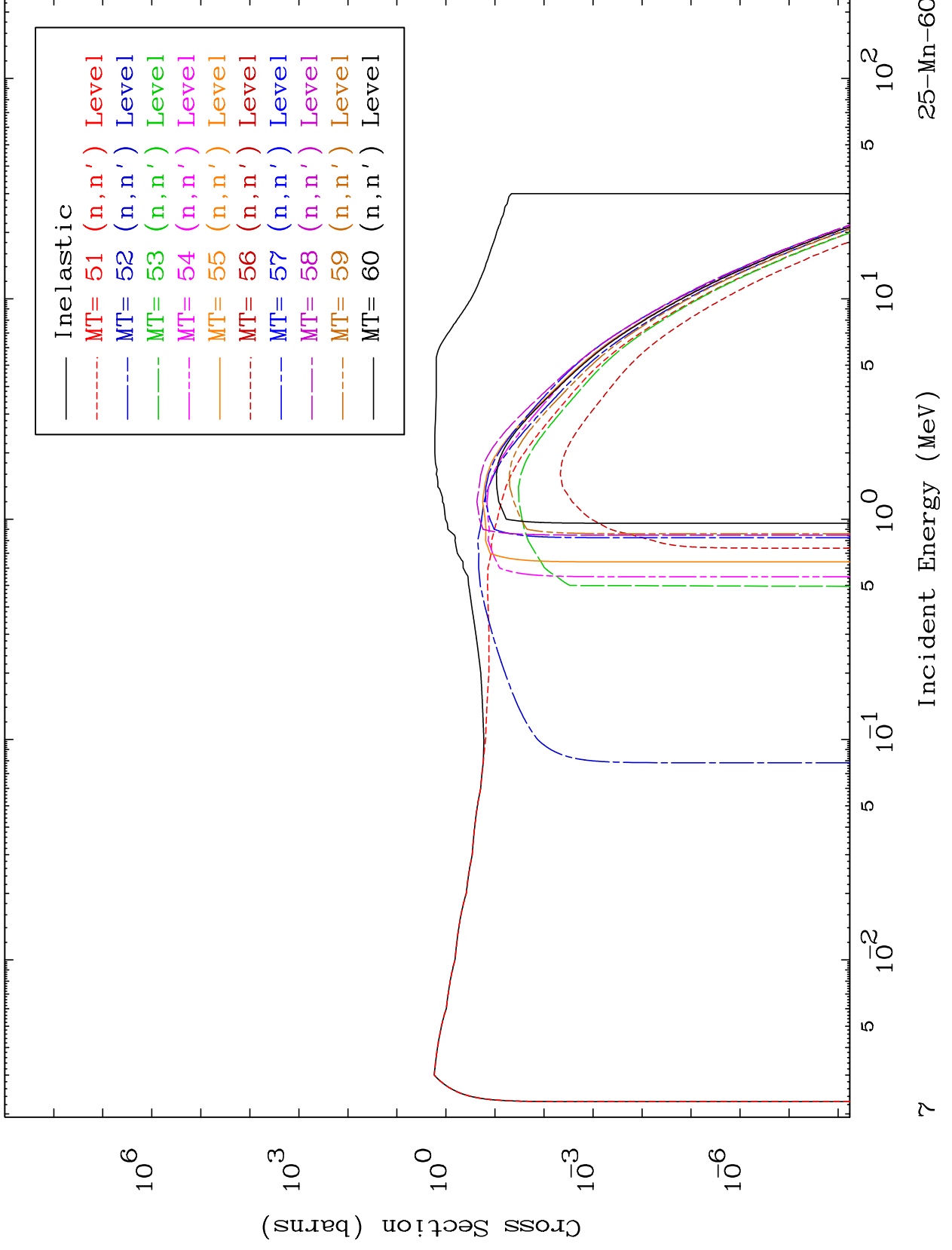
25-Mn-60

MAT 2541

(n,n') Level

293 Kelvin Cross Sections

25-Mn-60



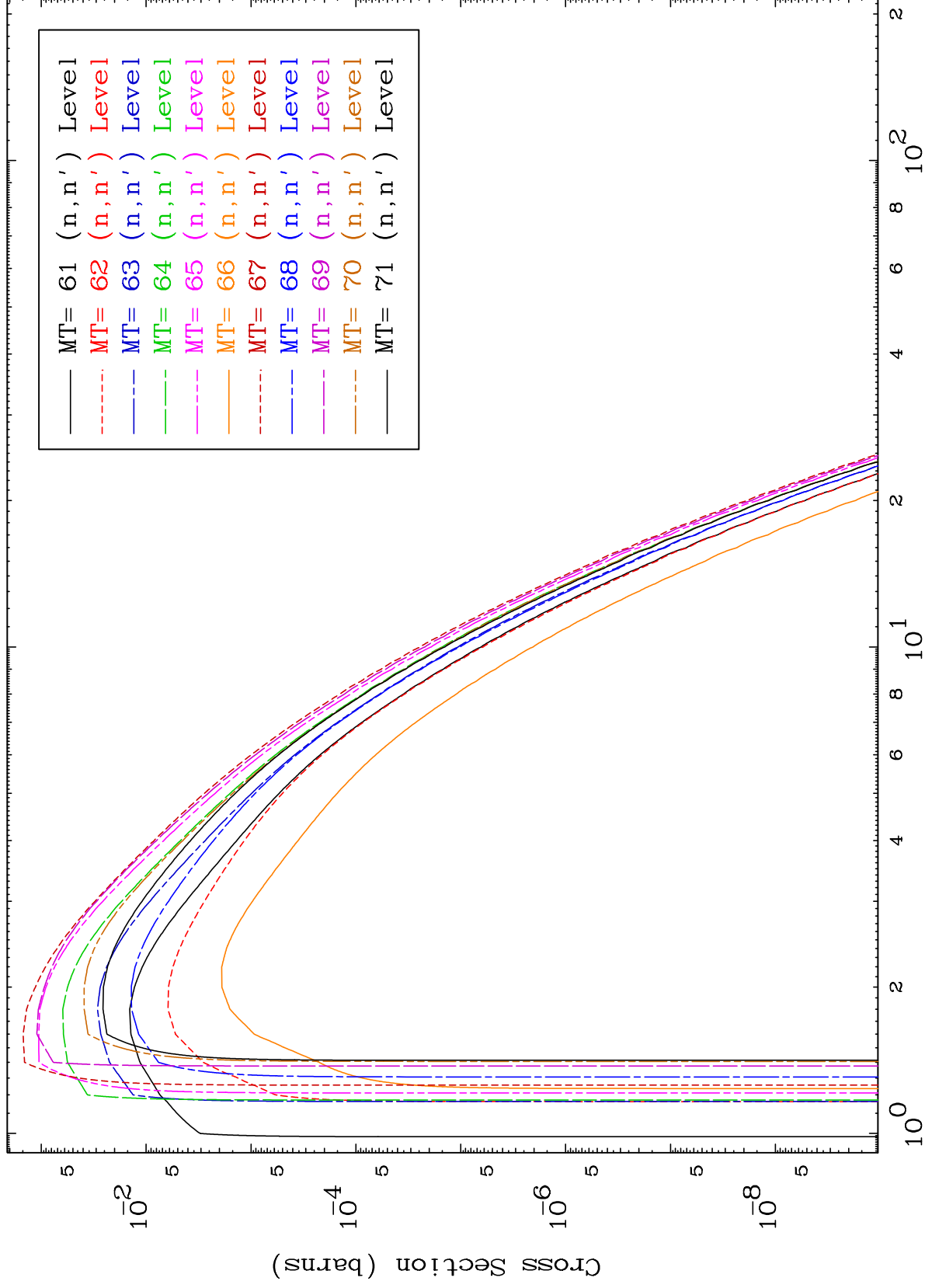


MAT 2541

(n,n') Level

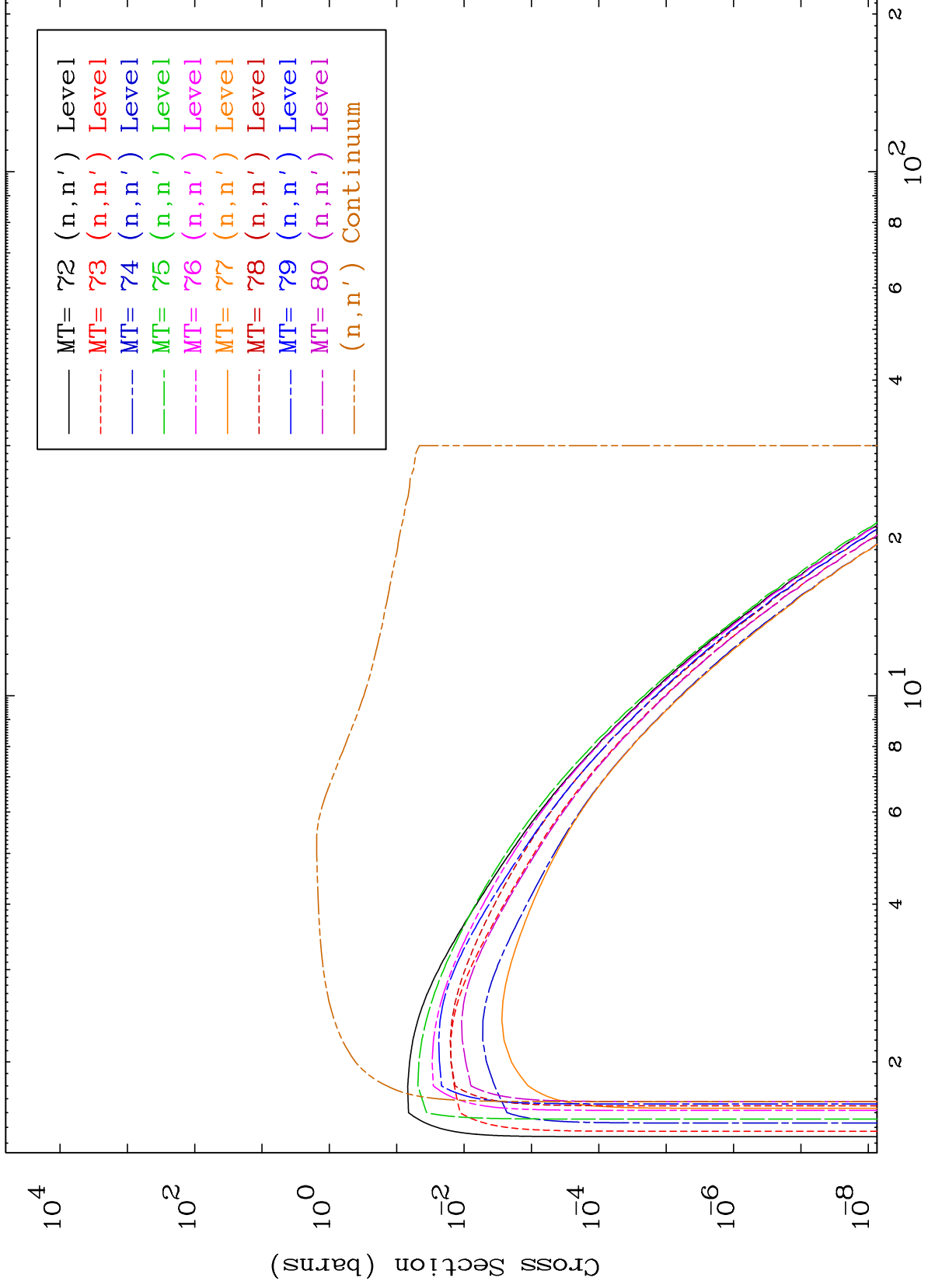
25-Mn-60

293 Kelvin Cross Sections



Incident Energy (MeV)

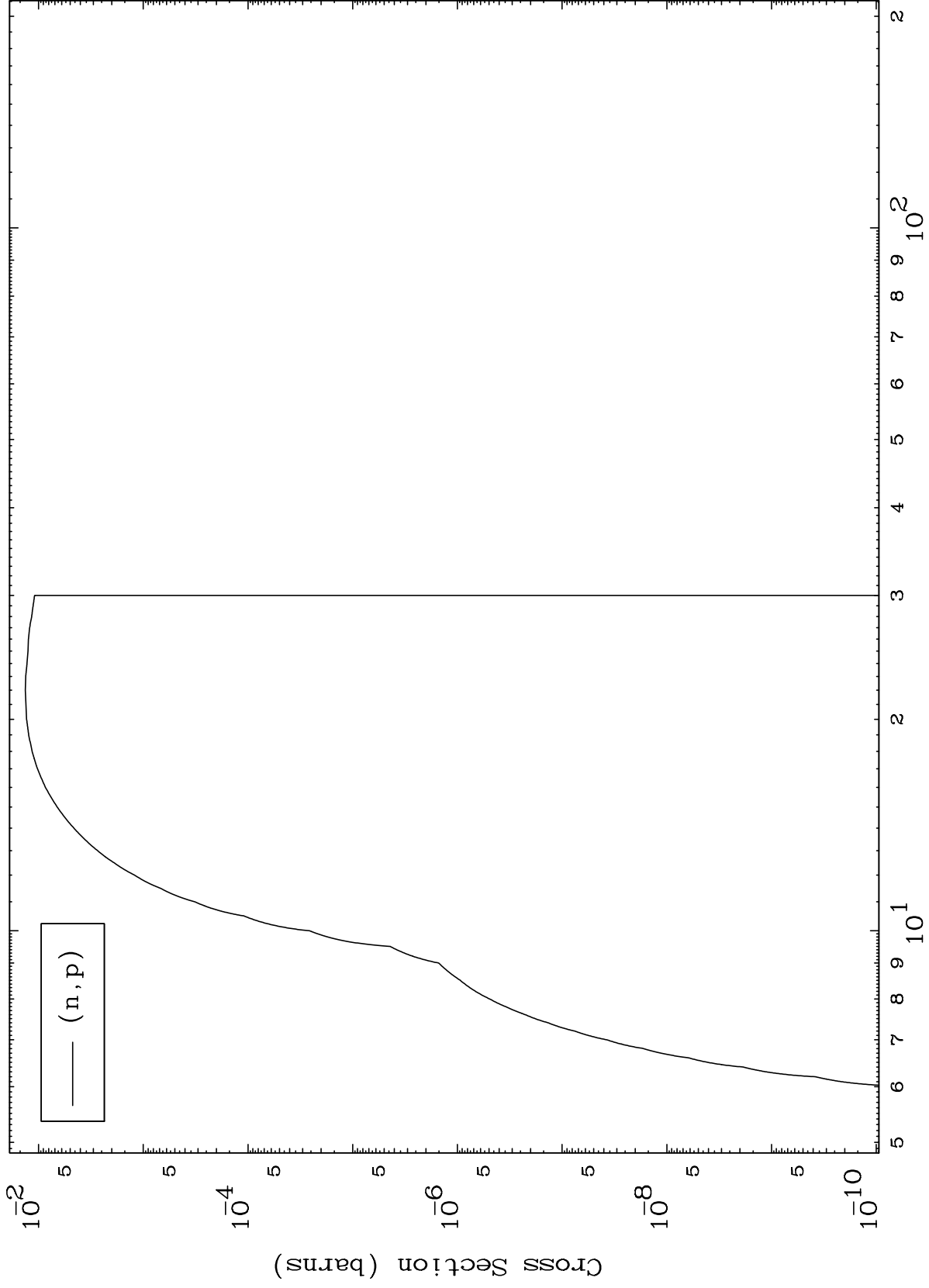
25-Mn-60



MAT 2541

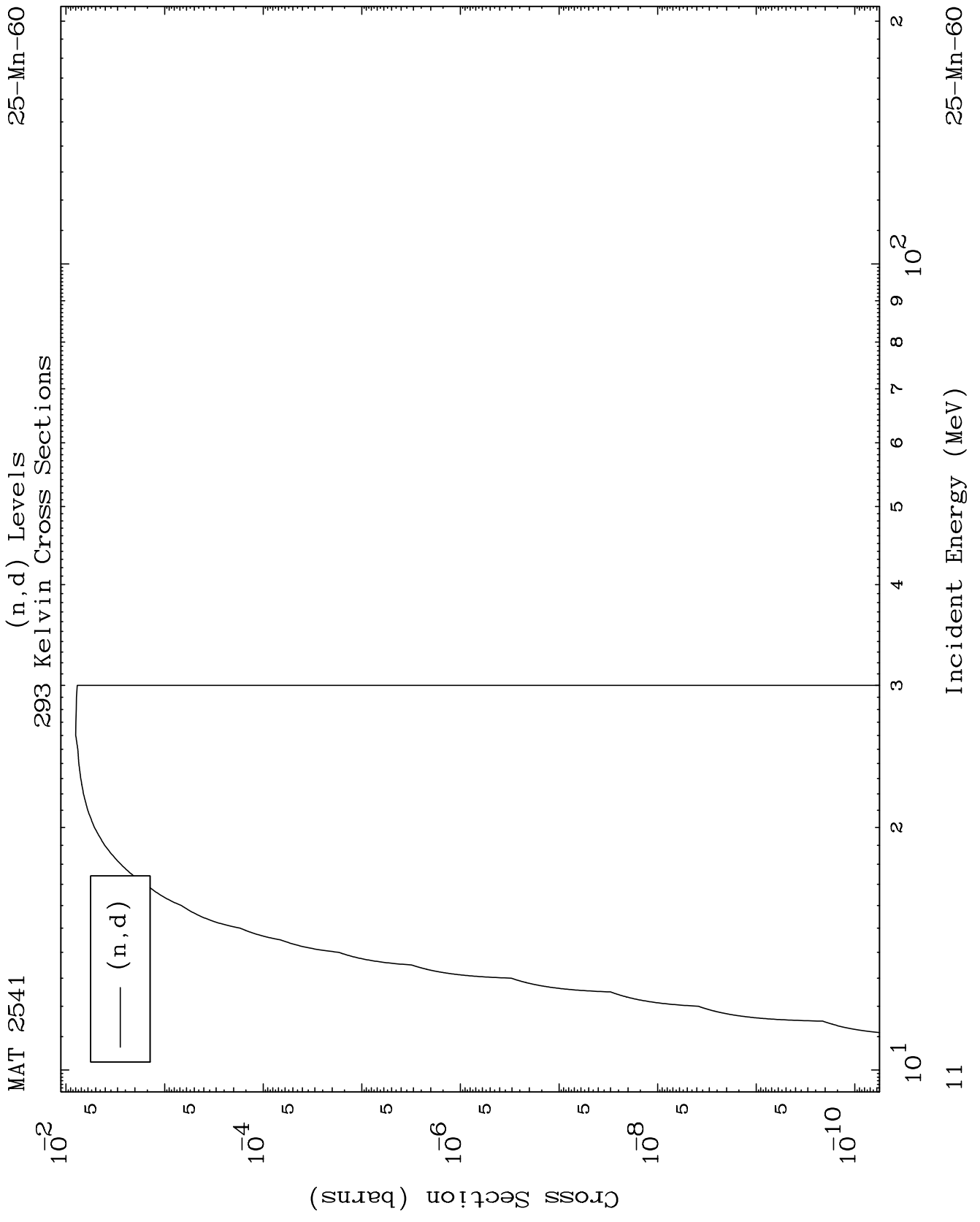
(n,p) Levels  
293 Kelvin Cross Sections

25-Mn-60



Incident Energy (MeV)

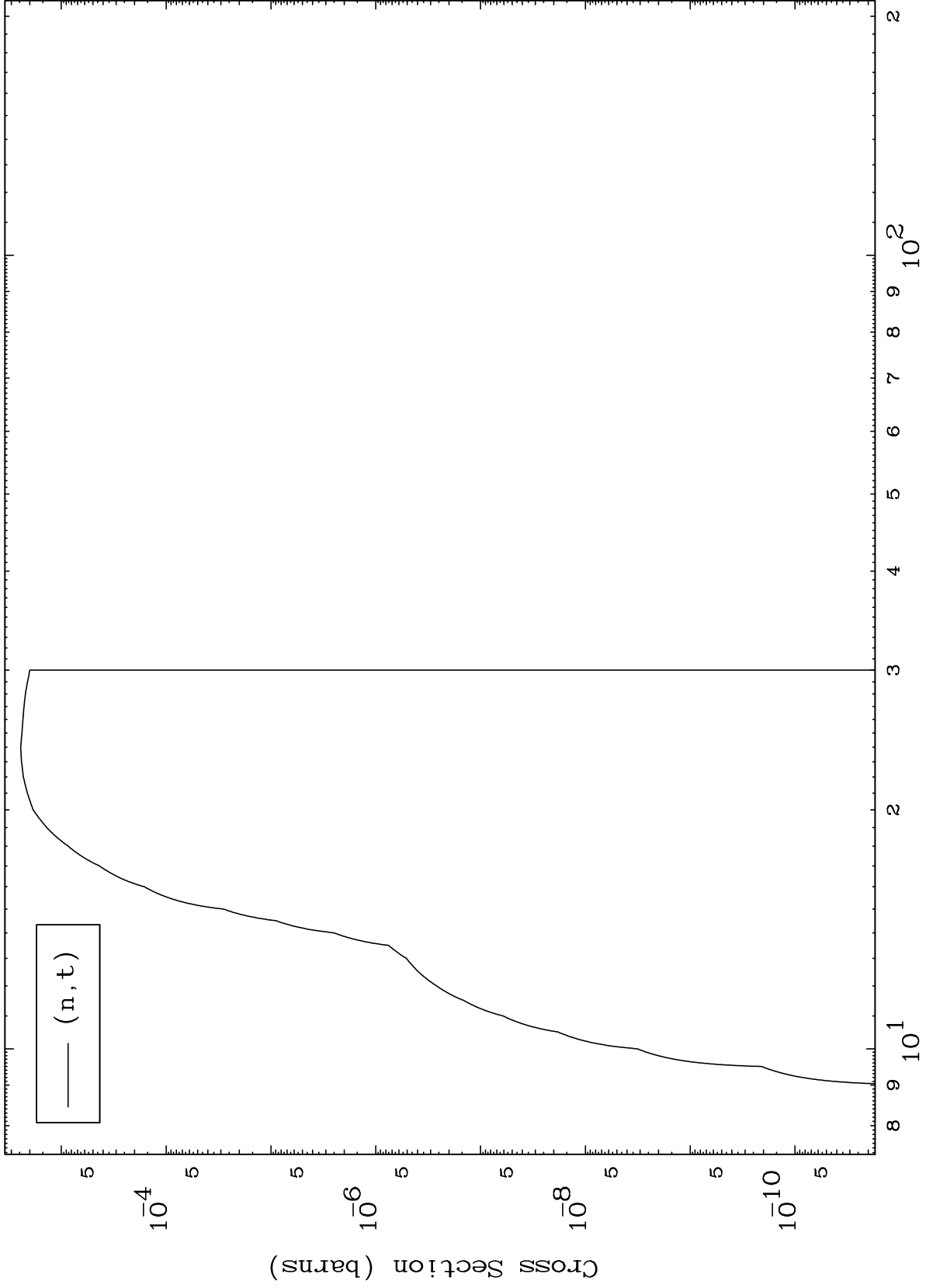
25-Mn-60



MAT 2541

(n,t) Levels  
293 Kelvin Cross Sections

25-Mn-60



12

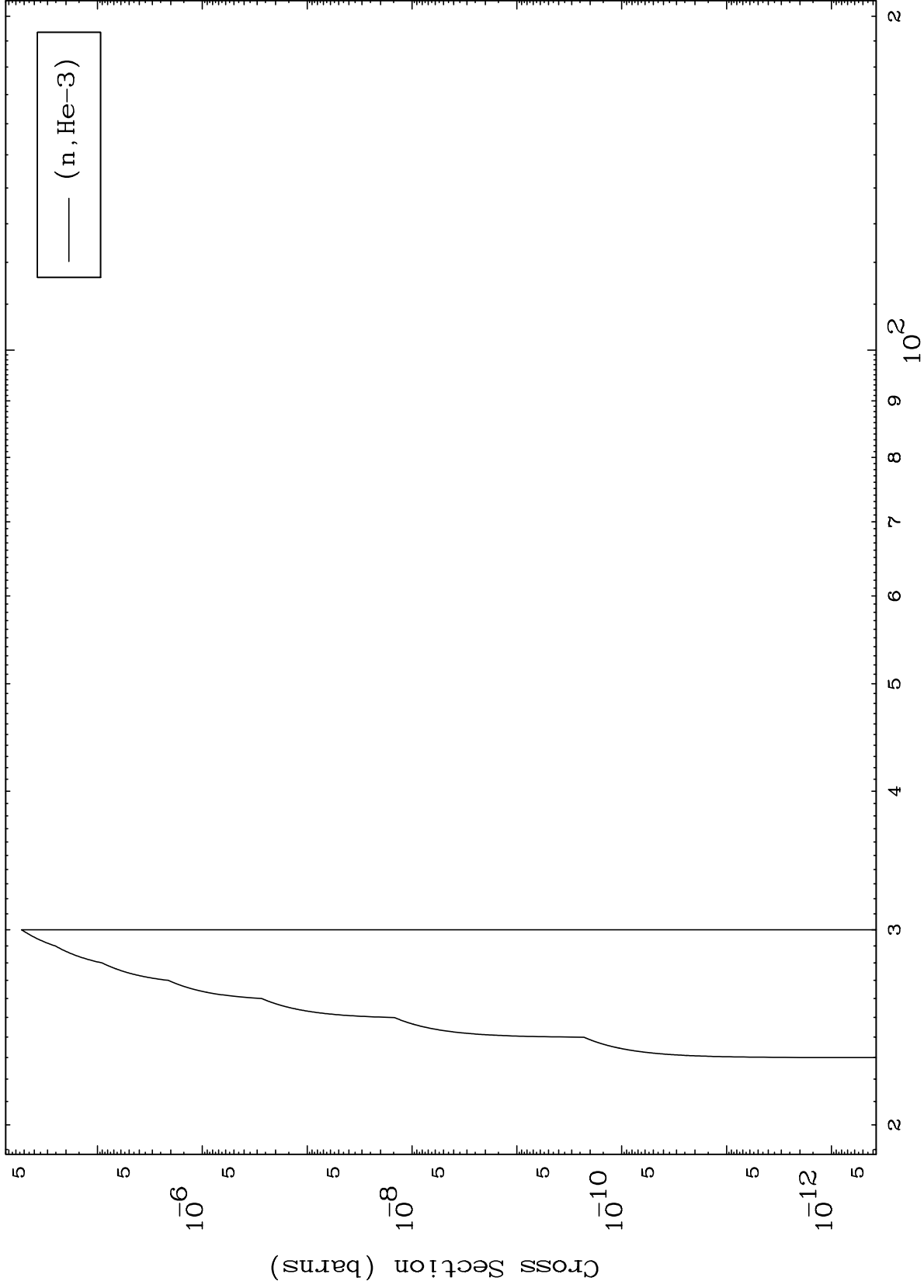
Incident Energy (MeV)

25-Mn-60

MAT 2541

(n,He3) Levels  
293 Kelvin Cross Sections

25-Mn-60



13

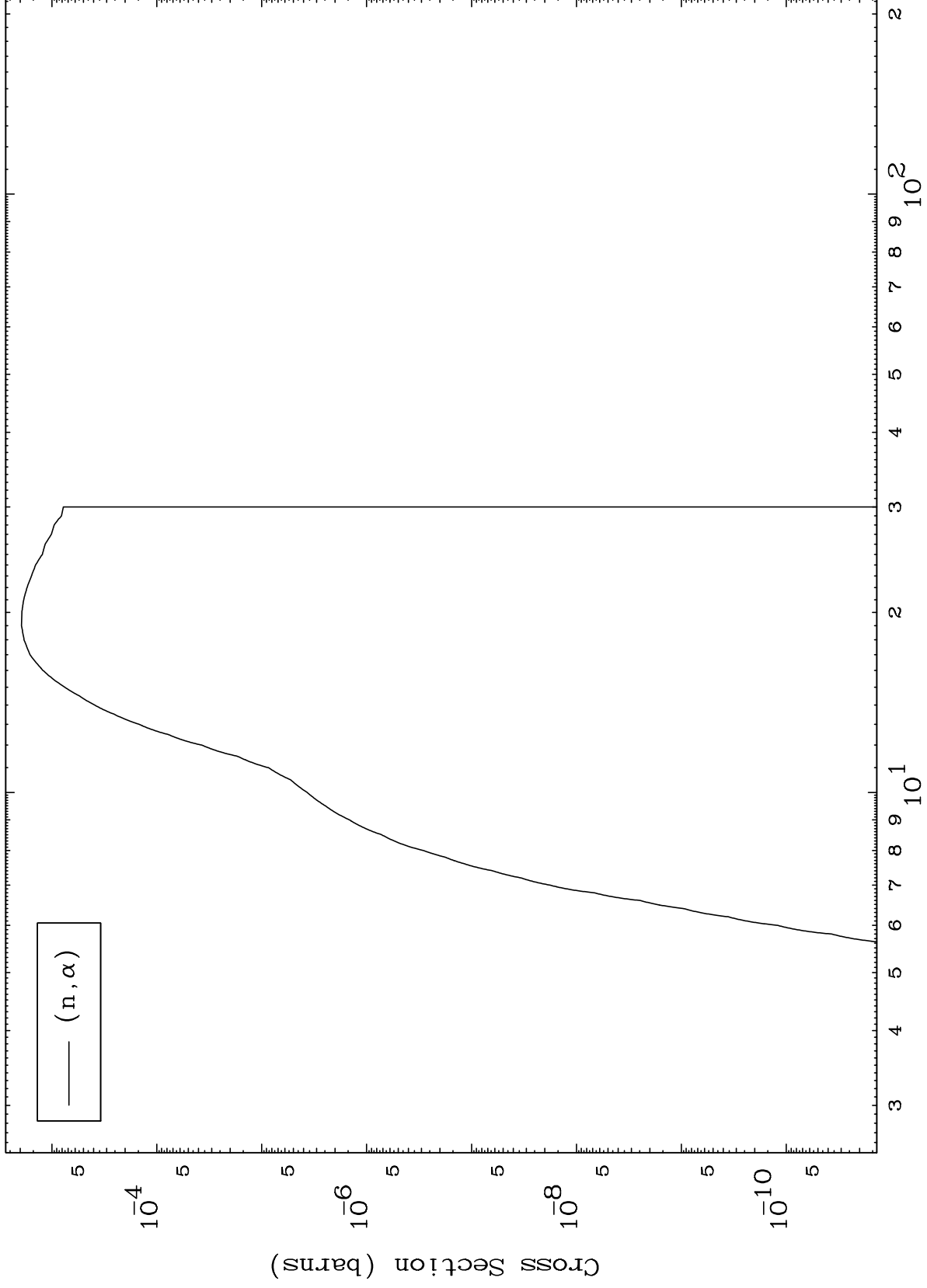
Incident Energy (MeV)

25-Mn-60

MAT 2541

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

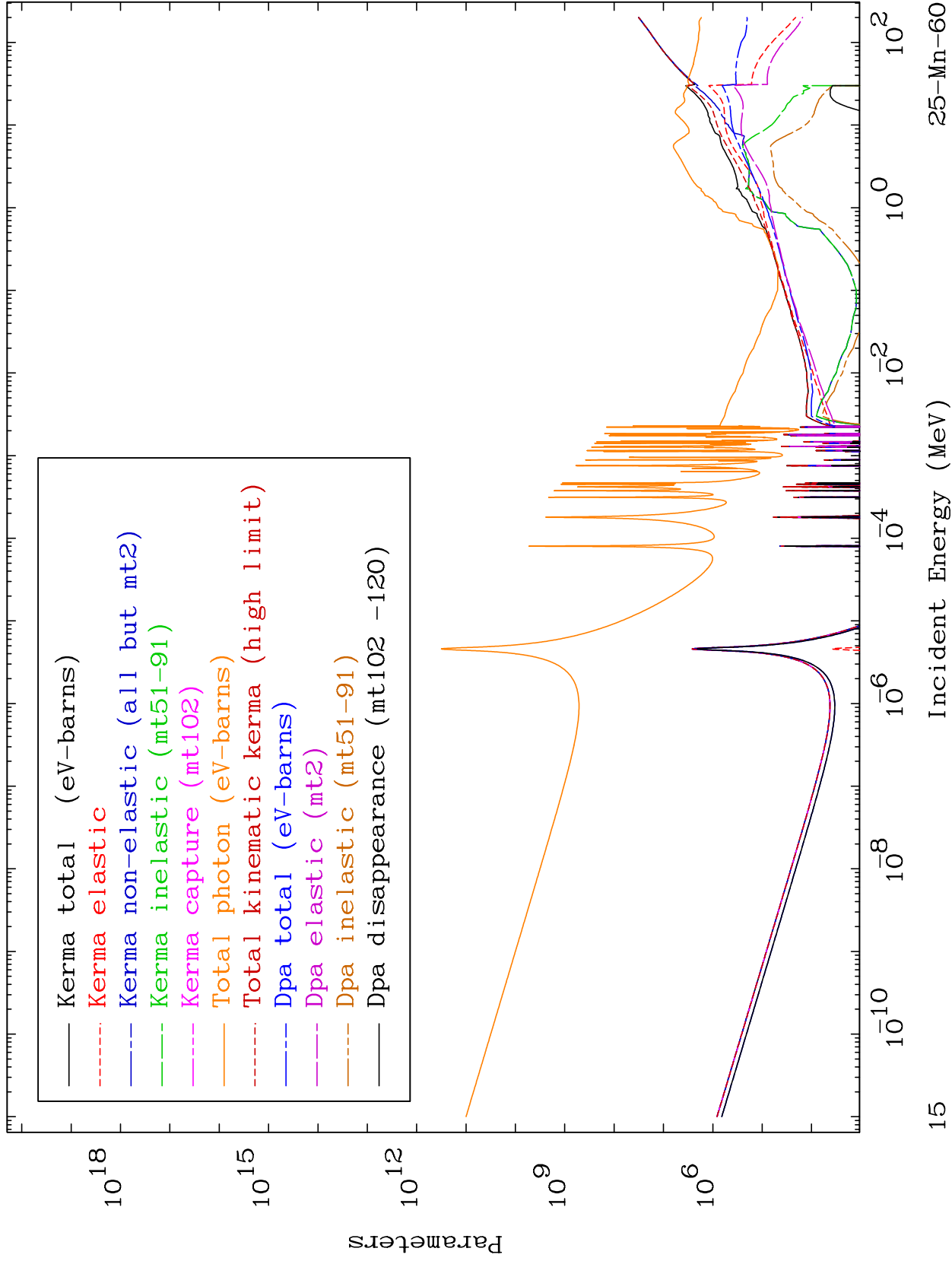
25-Mn-60



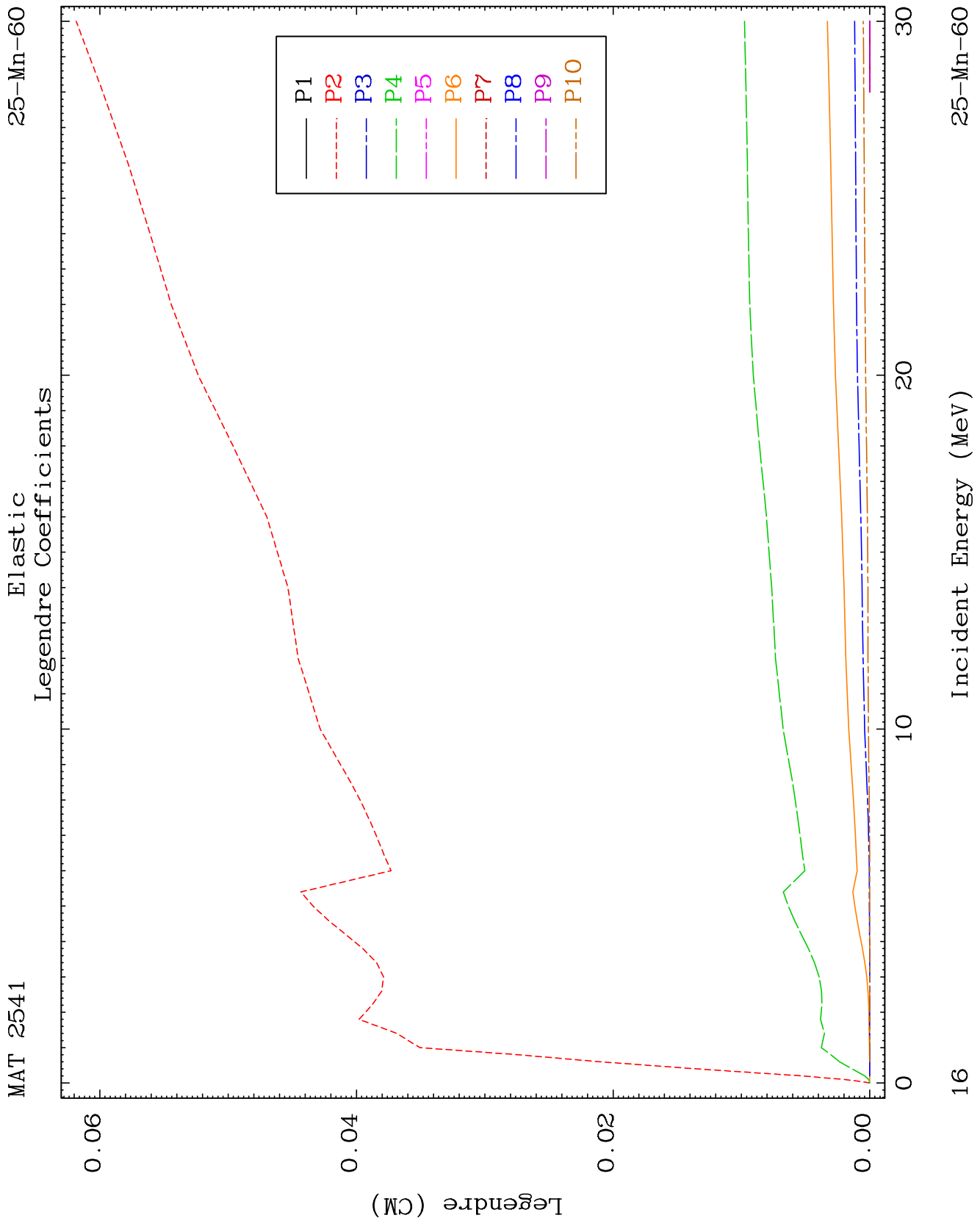
14

Incident Energy (MeV)

25-Mn-60









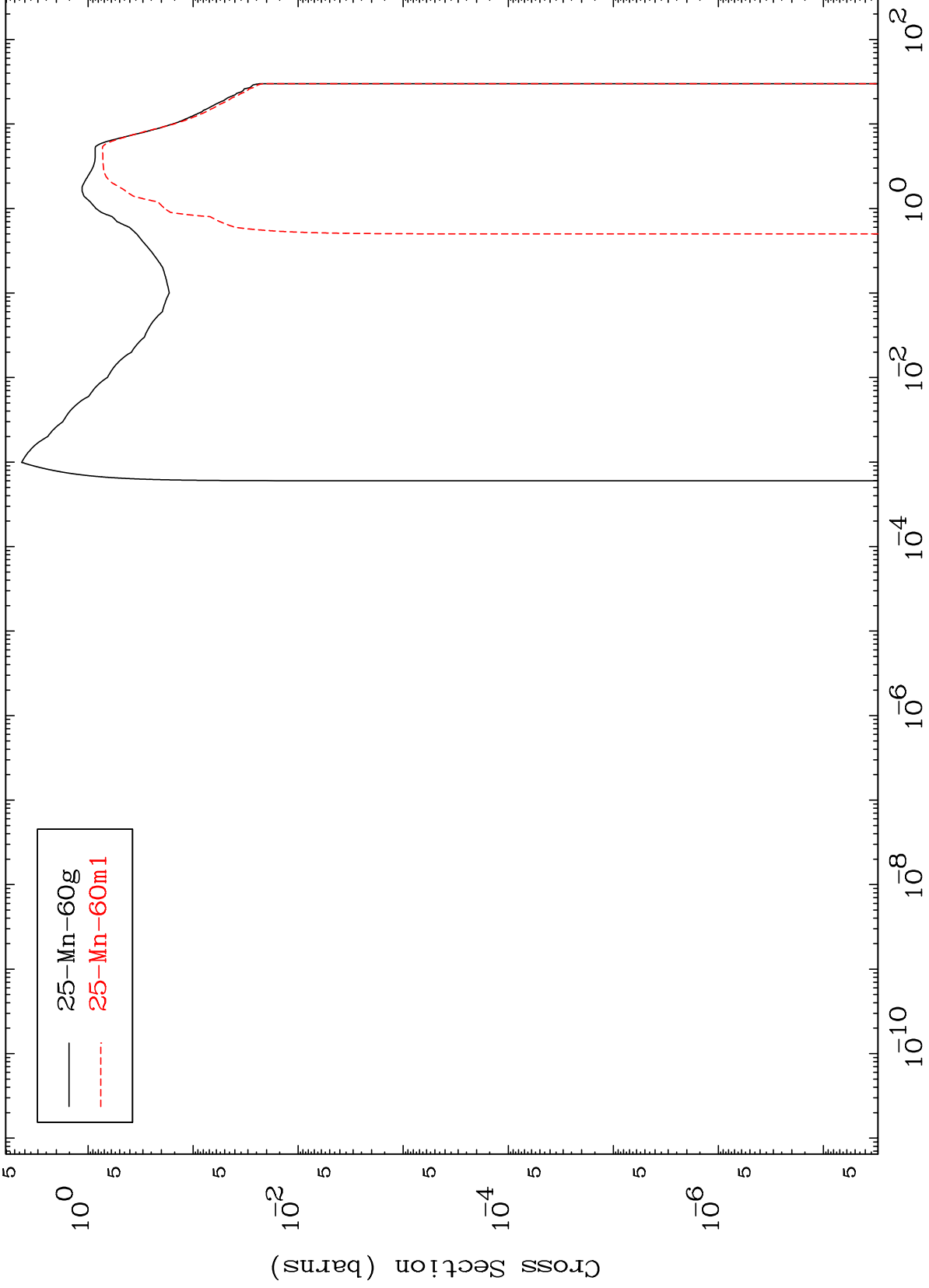


MAT 2541

Inelastic

25-Mn-60

Radionuclide Production Cross Section

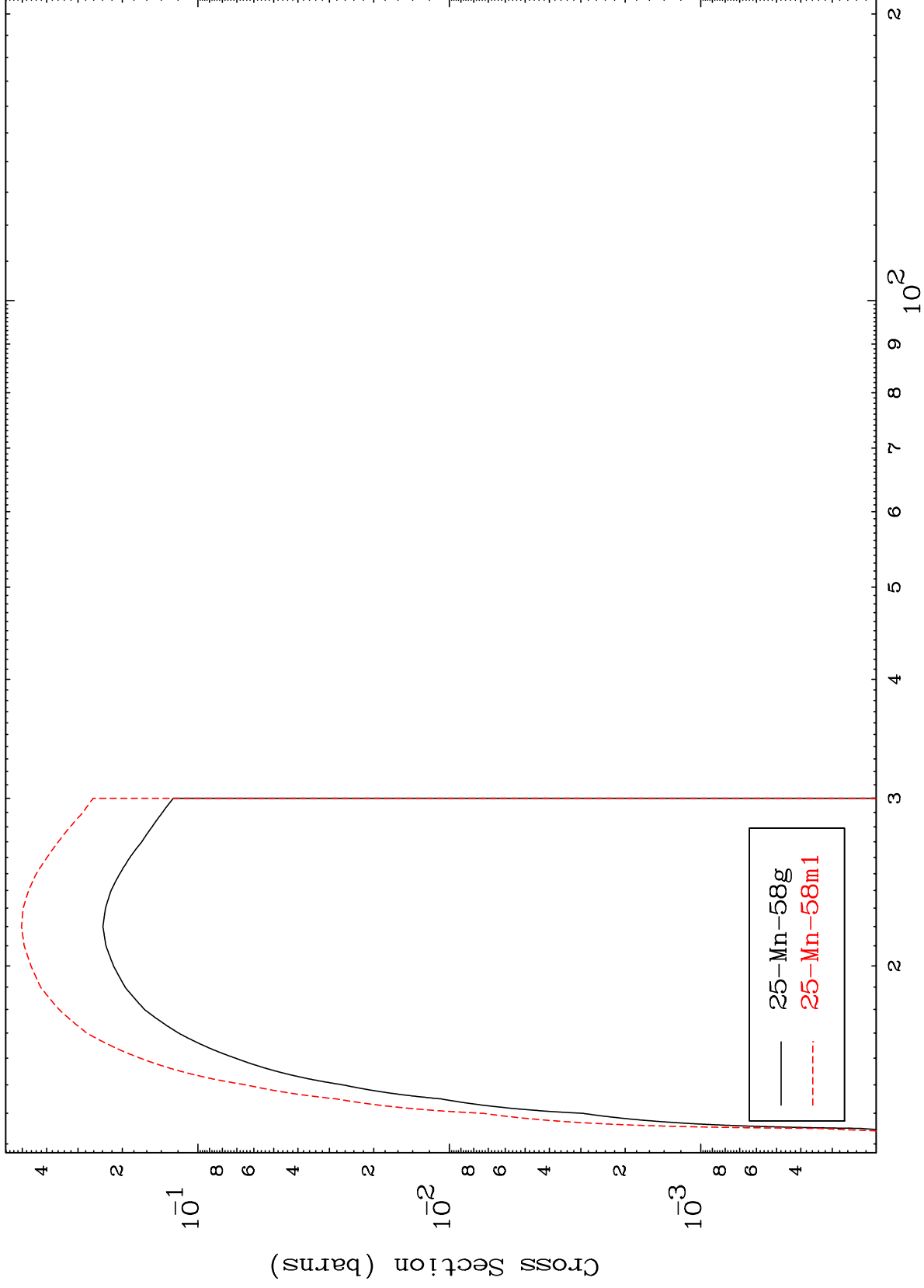


MAT 2541

(n,3n)

<sup>25</sup>Mn-60

Radionuclide Production Cross Section



20

Incident Energy (MeV)

<sup>25</sup>Mn-60