

Program EVALPLOT
(Version 2018-1)

by

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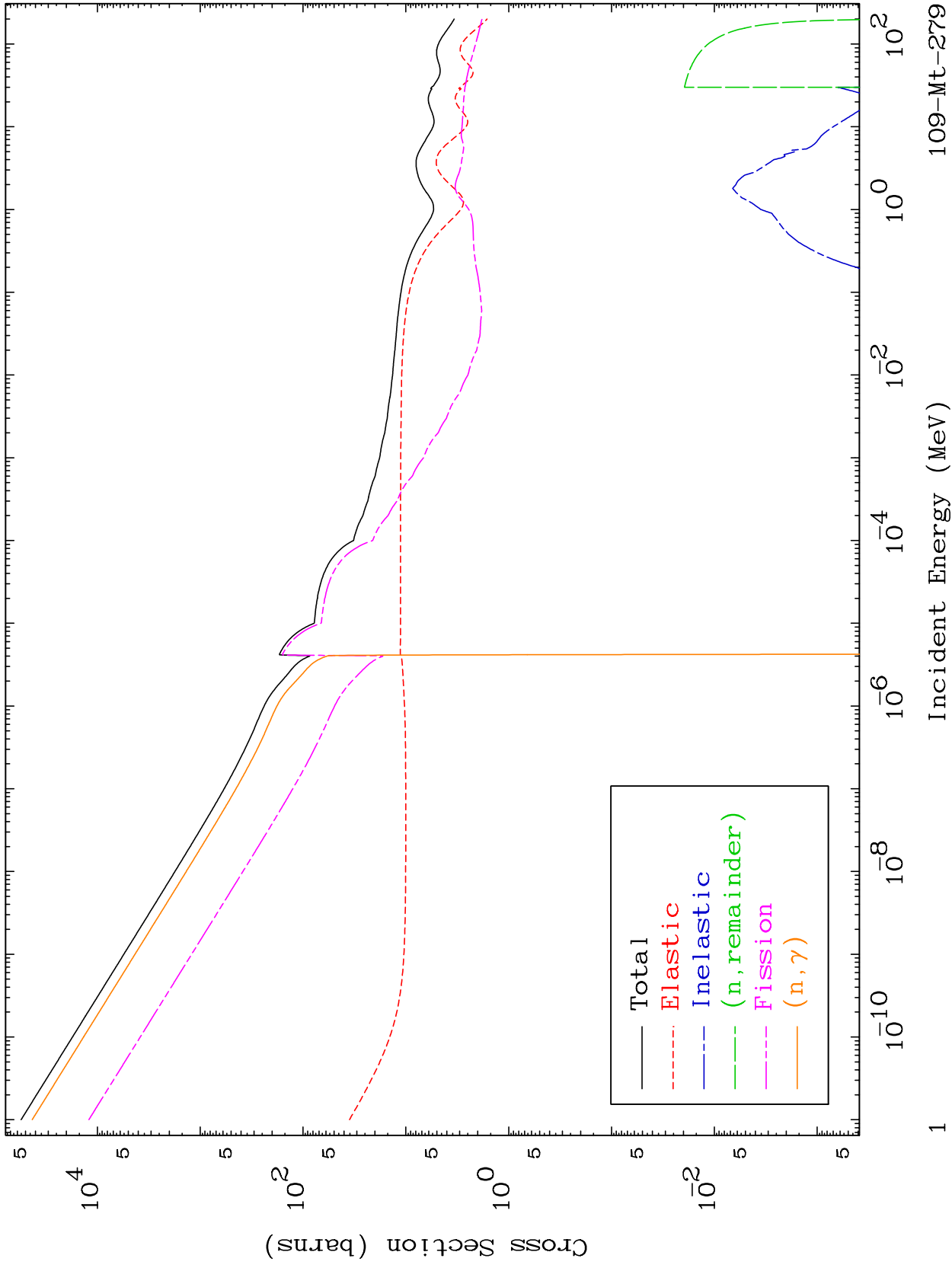
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 979

Major
293 Kelvin Cross Sections

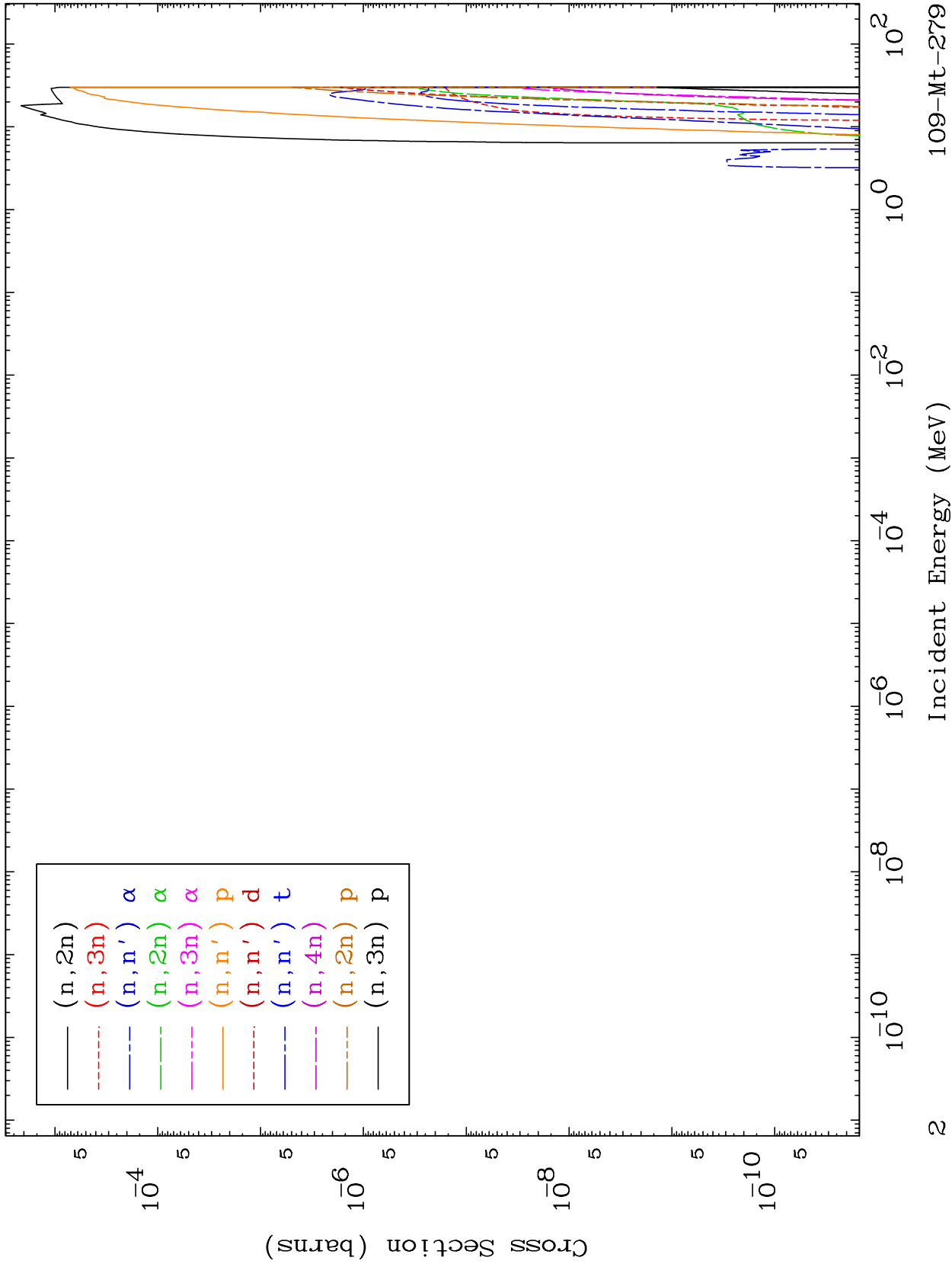
109-Mt-279



MAT 979

Neutron Production
293 Kelvin Cross Sections

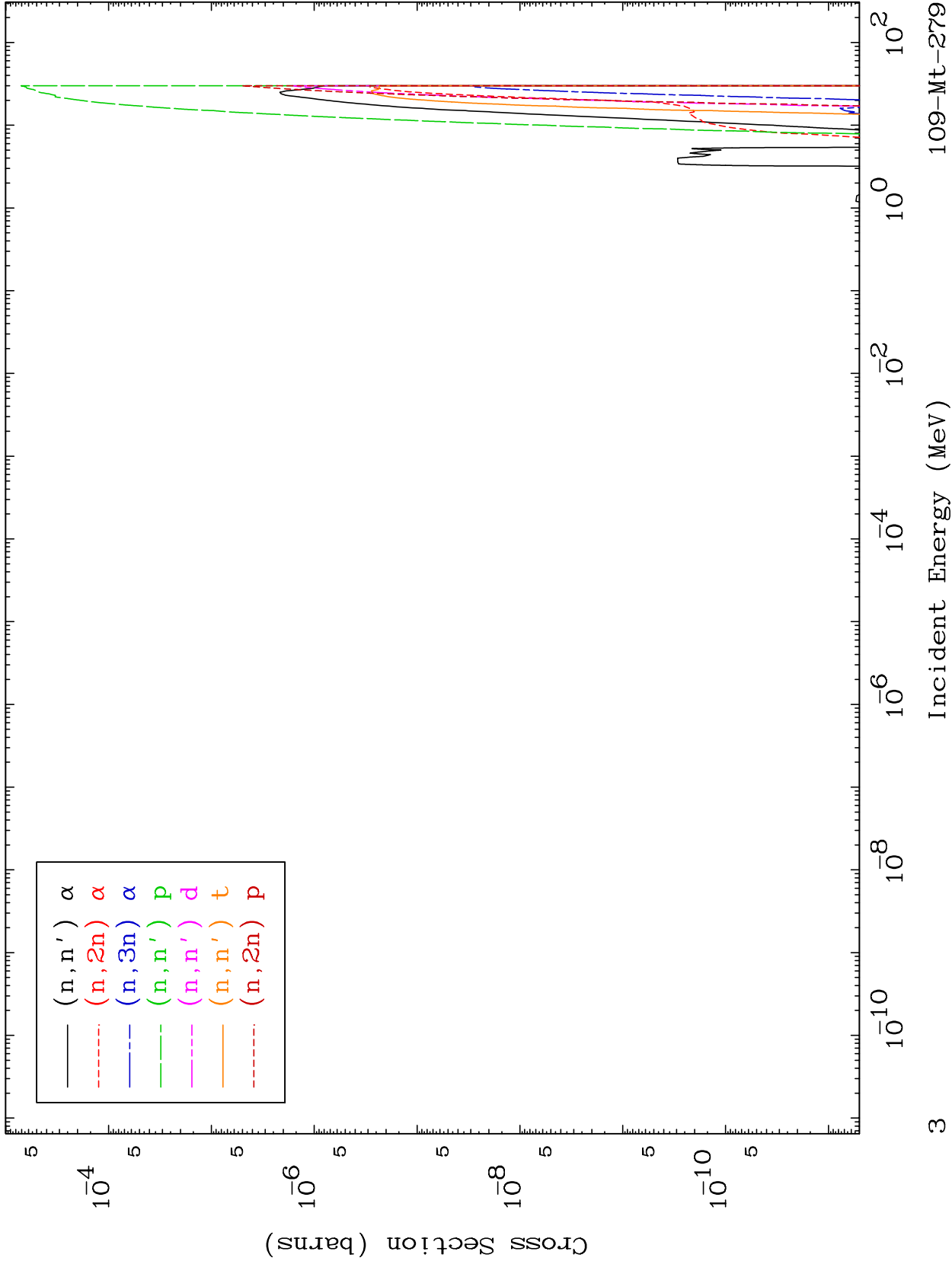
109-Mt-279

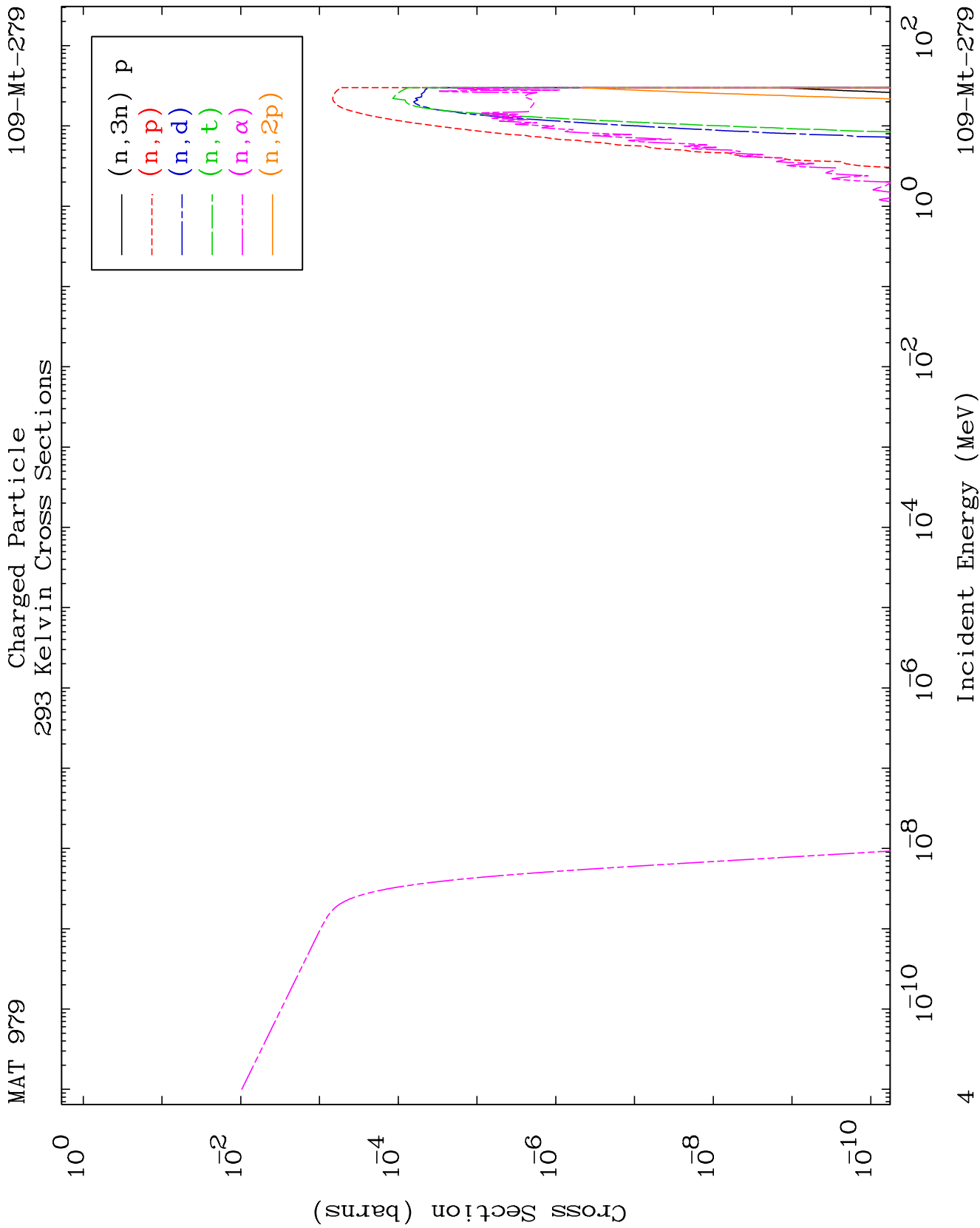


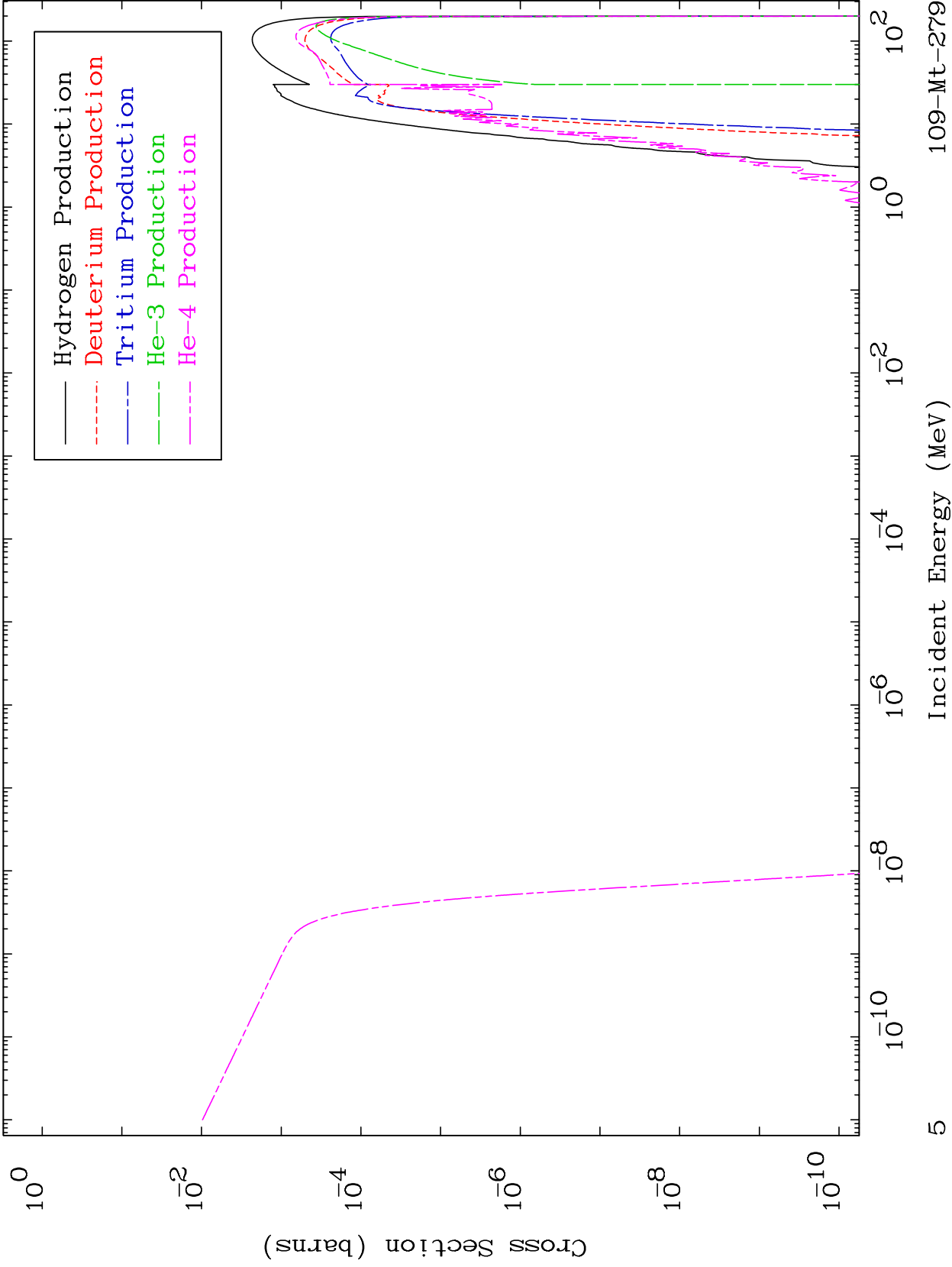
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Charged Particle
293 Kelvin Cross Sections

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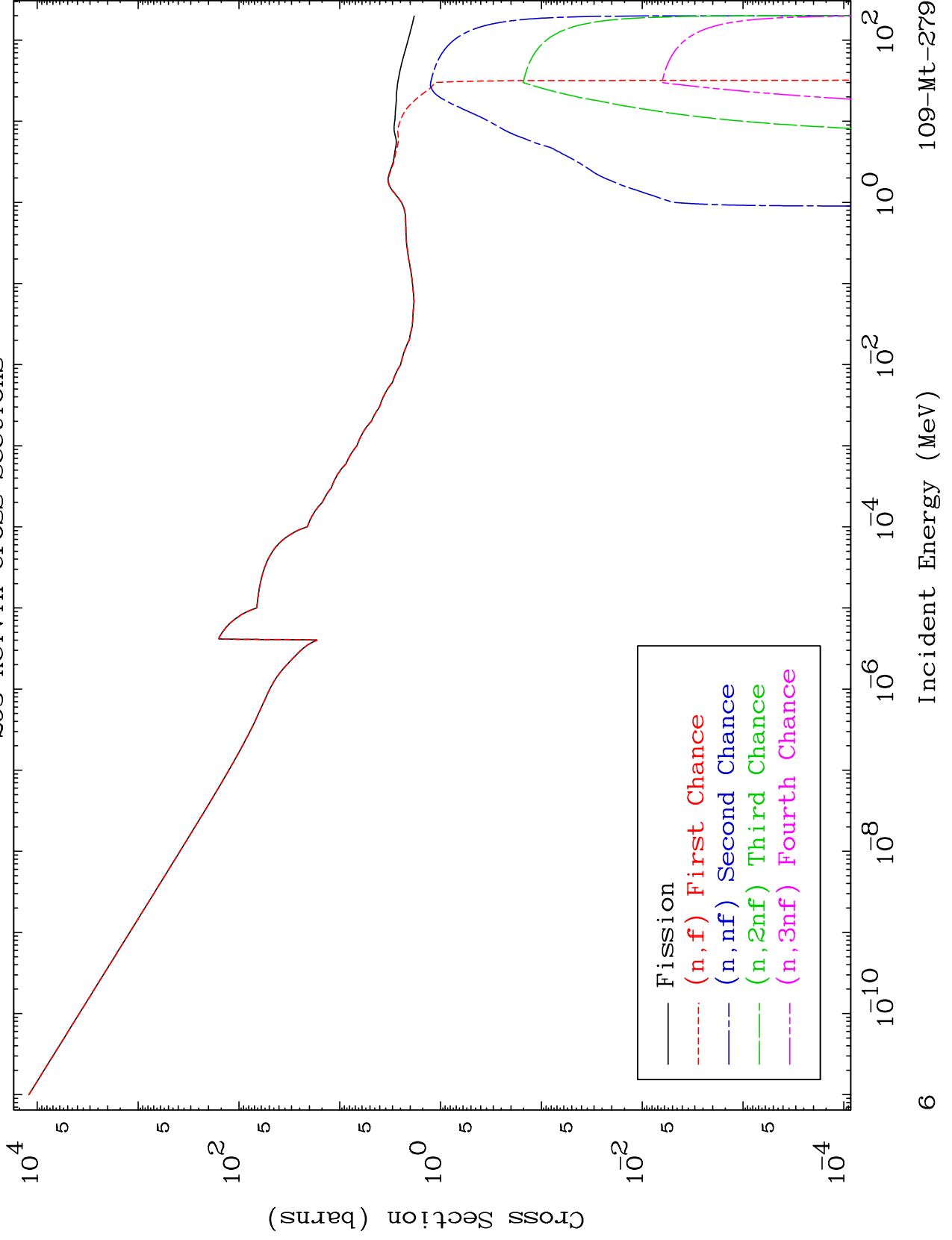




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Fission
293 Kelvin Cross Sections

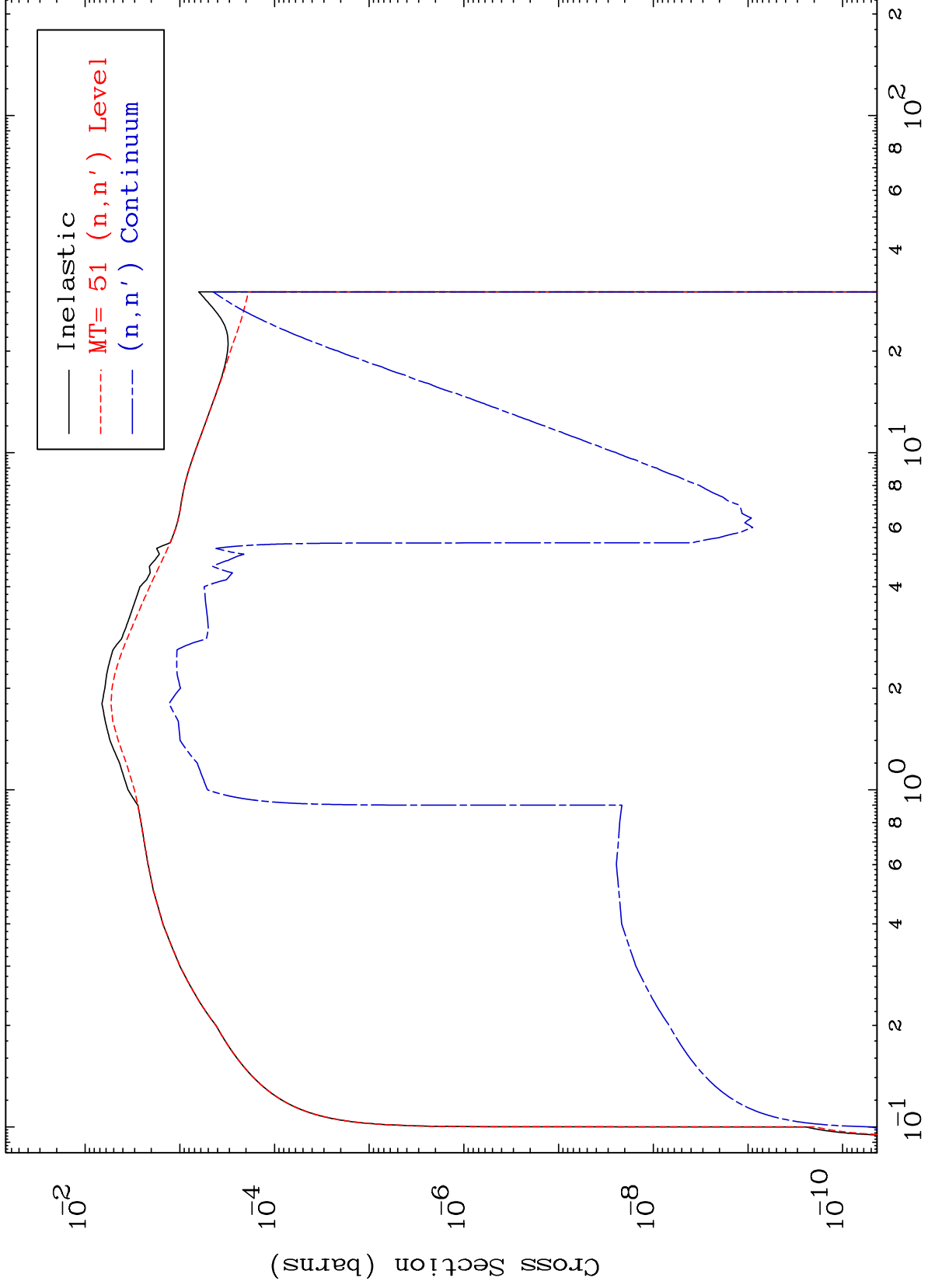
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(n,n') Level
293 Kelvin Cross Sections

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— Inelastic
- - - MT= 51 (n,n') Level
- - - (n,n') Continuum

Incident Energy (MeV)

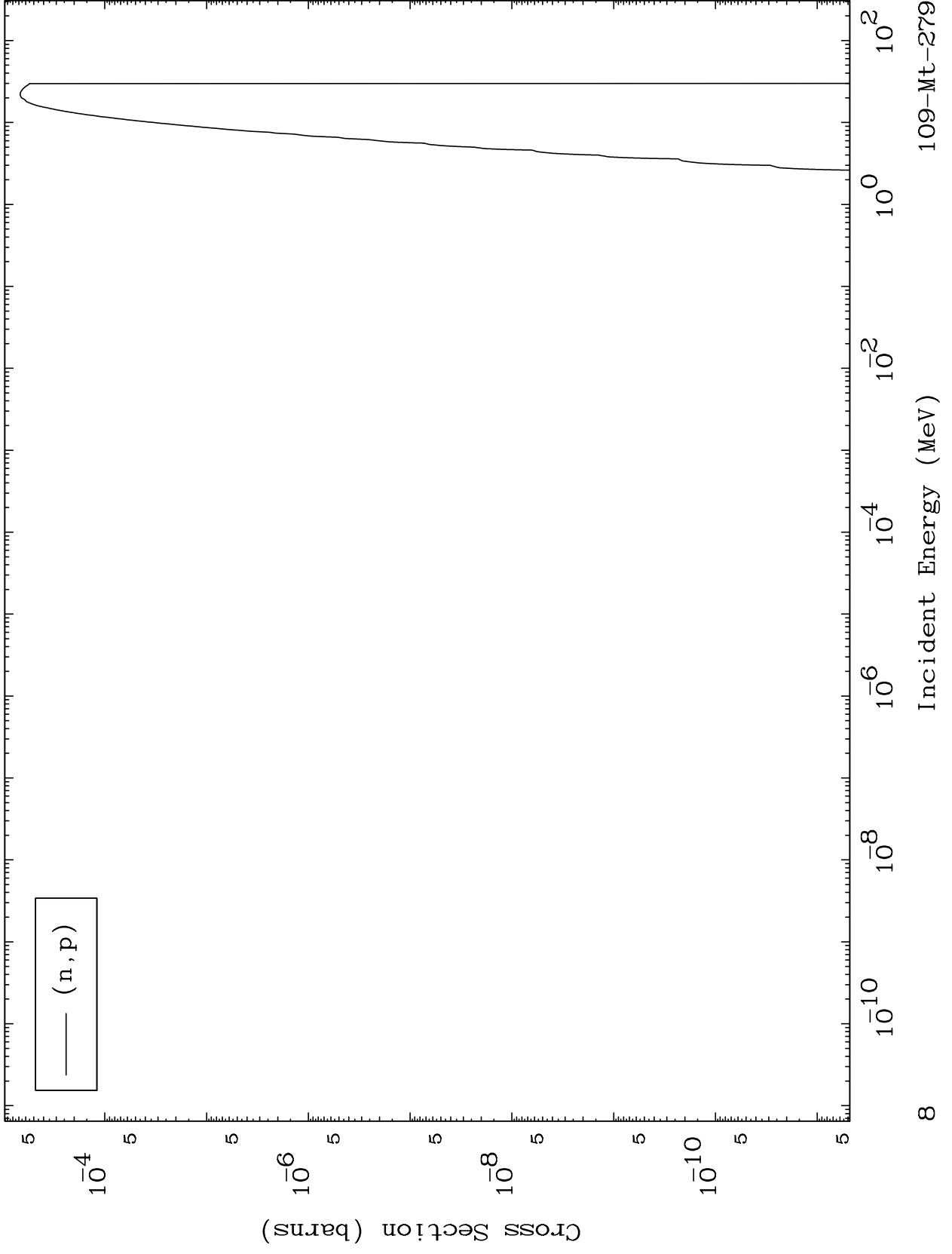
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(n,p) Levels
293 Kelvin Cross Sections

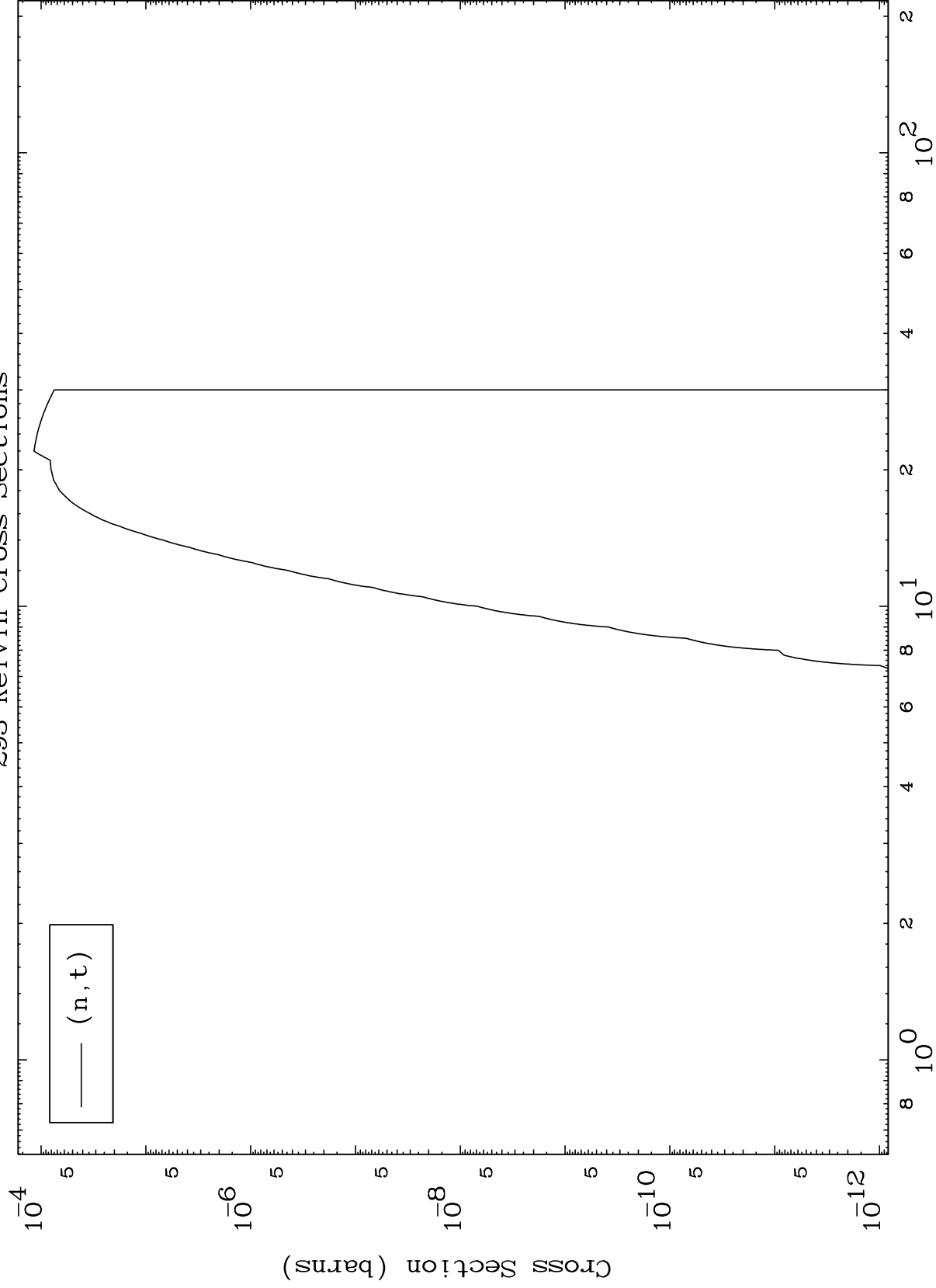
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(n,t) Levels
293 Kelvin Cross Sections

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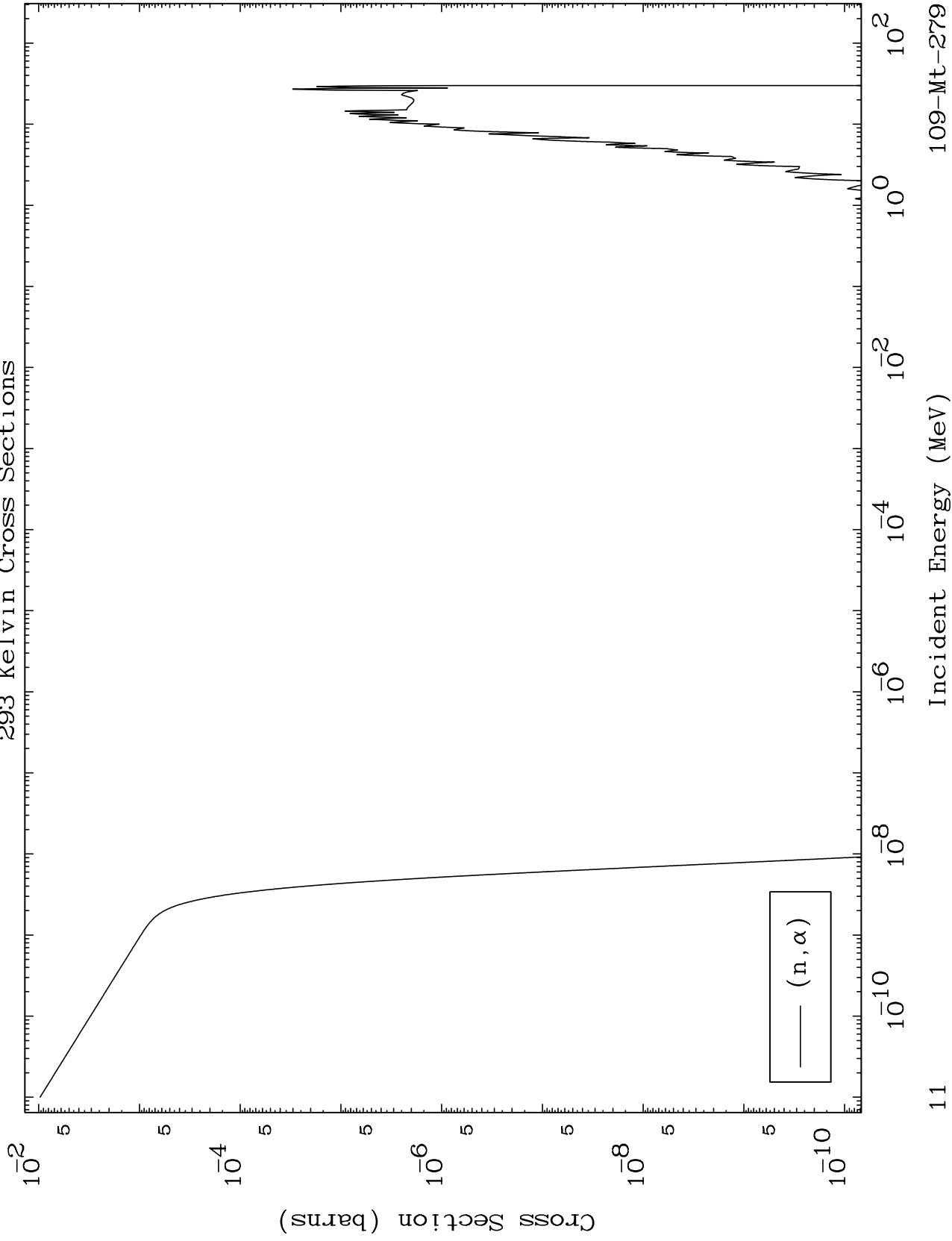
Incident Energy (MeV)

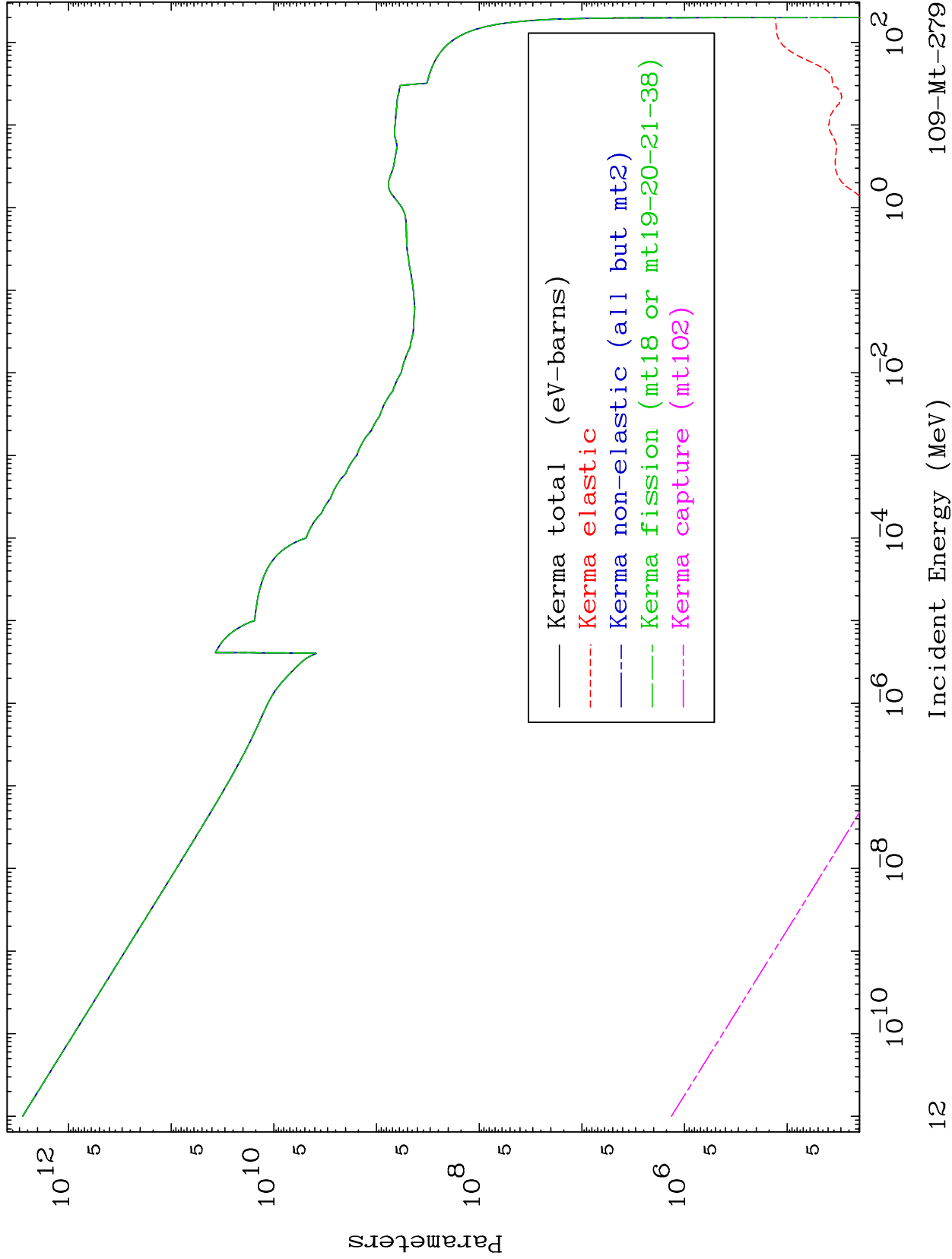
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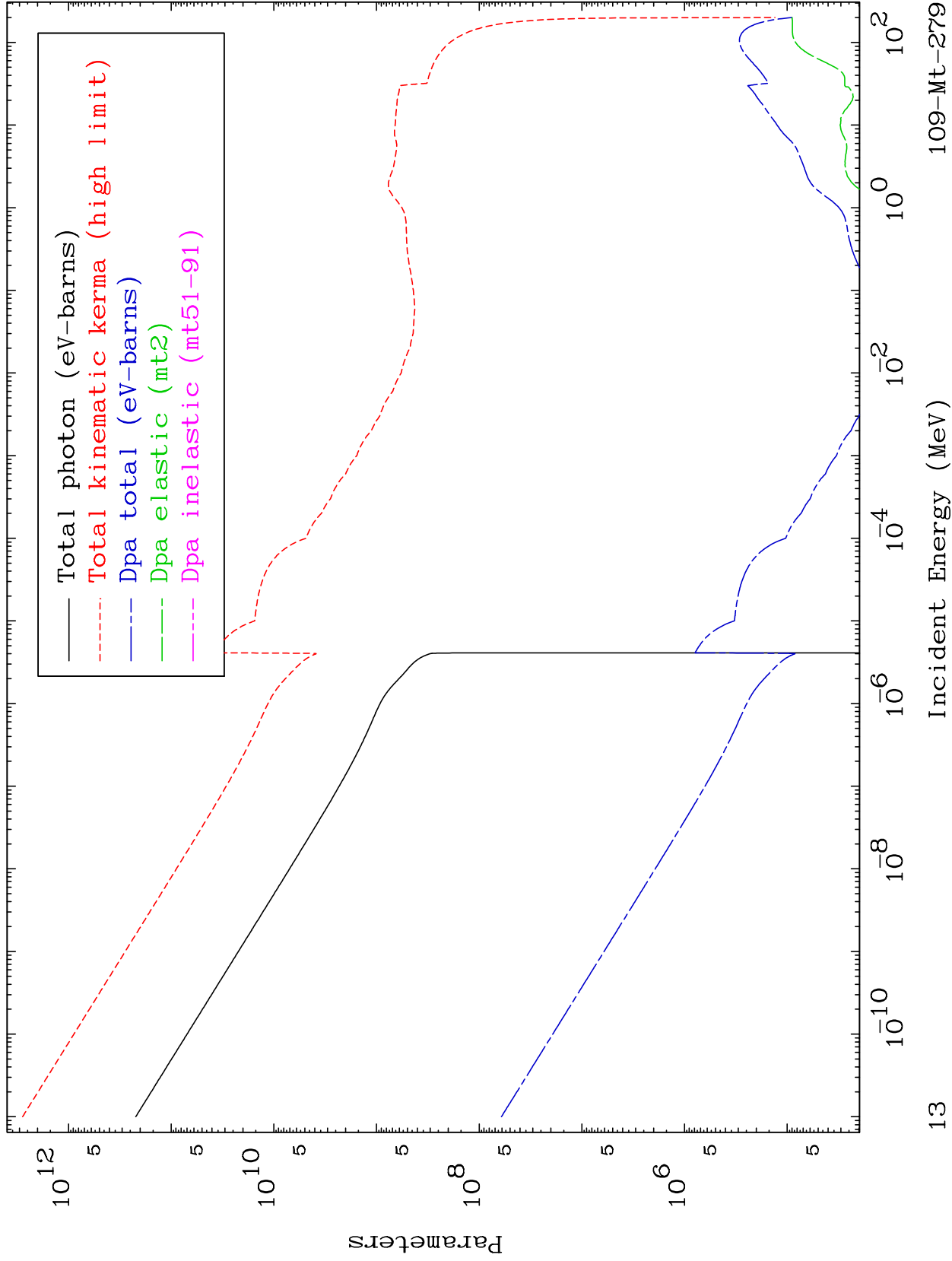
MAT 979

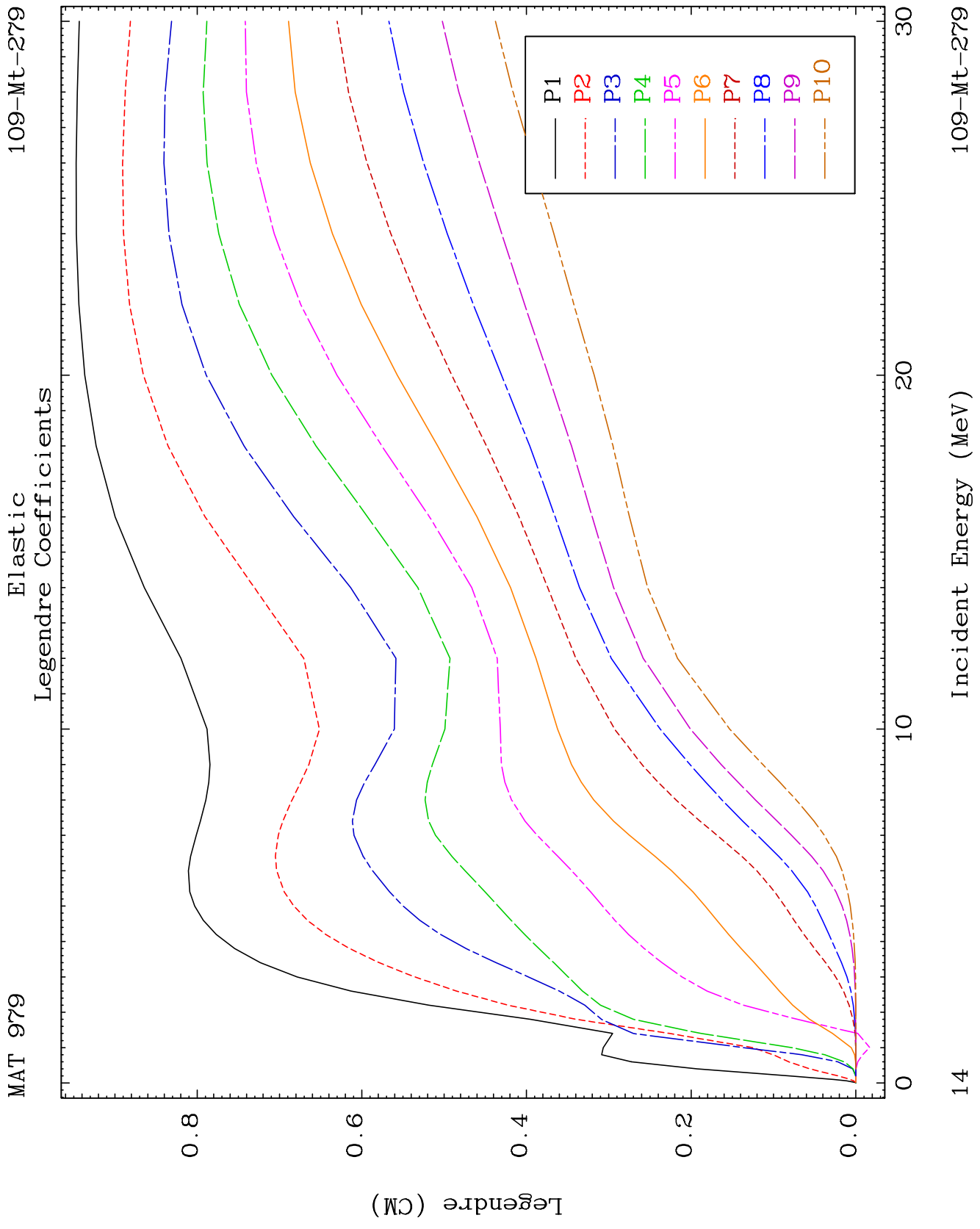
(n, α) Levels
293 Kelvin Cross Sections

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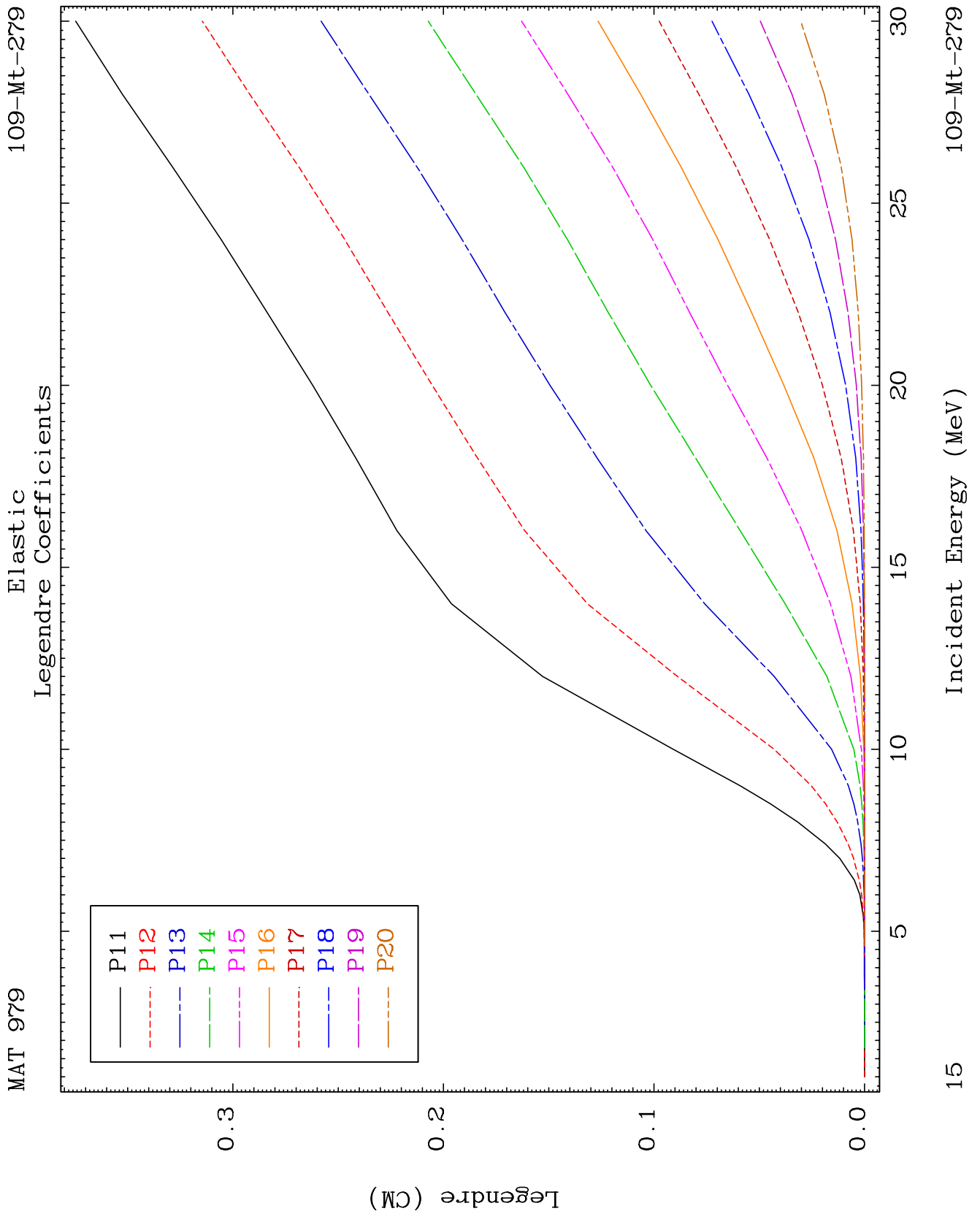




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Incident Energy (MeV)

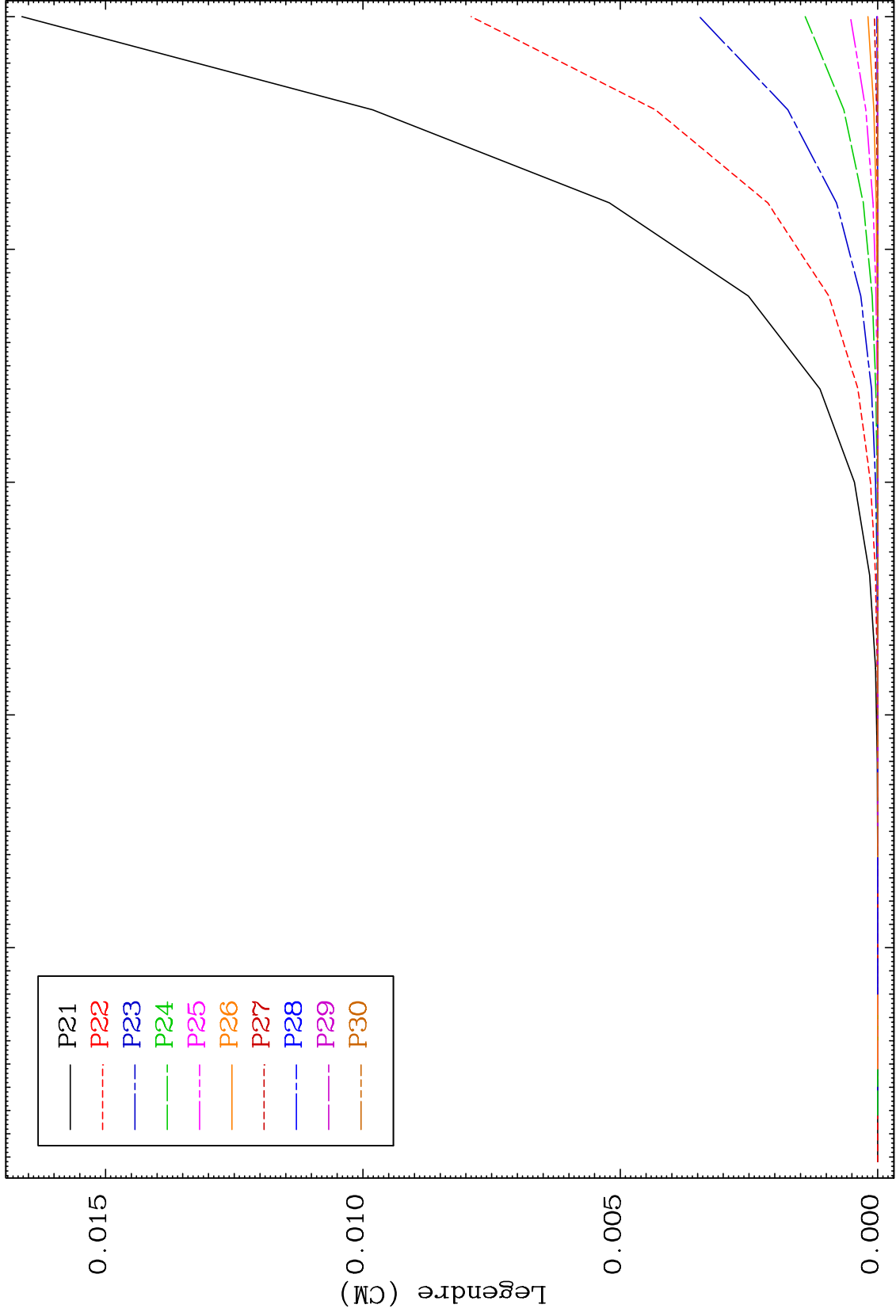
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Elastic Legendre Coefficients

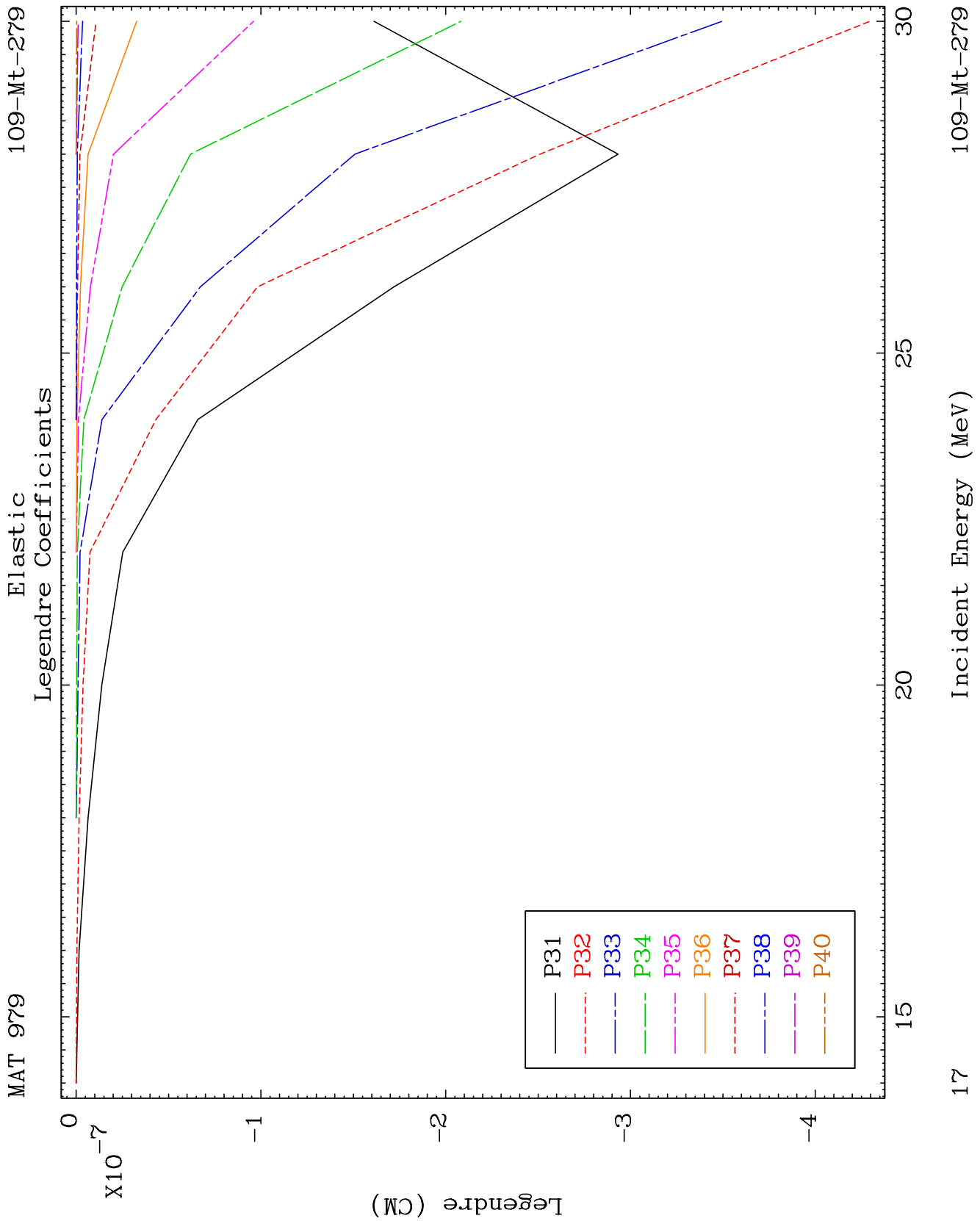
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Incident Energy (MeV)

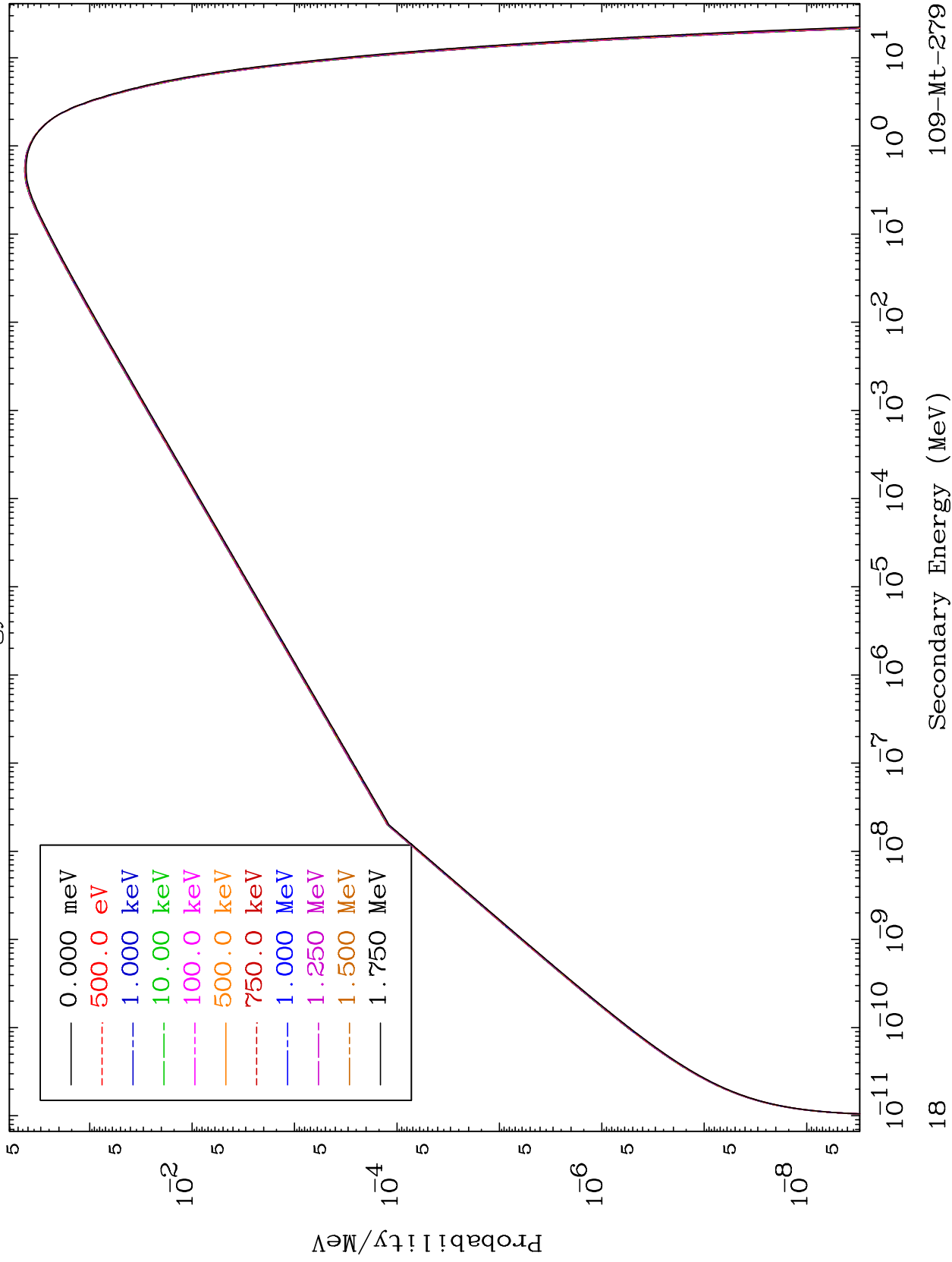
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Fission Energy Distribution

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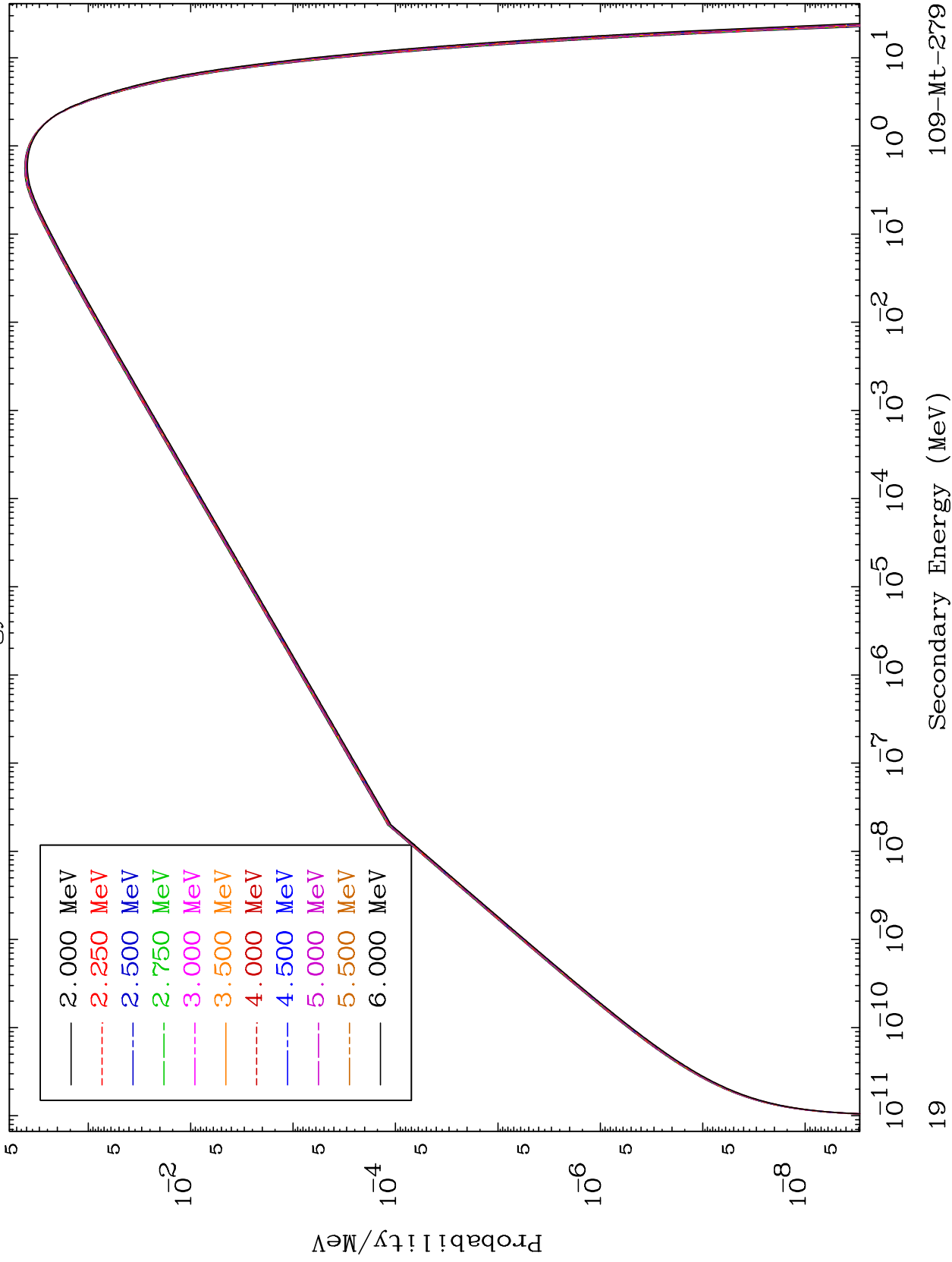
Secondary Energy (MeV)

109-Mt-279

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Fission Energy Distribution

109-Mt-279



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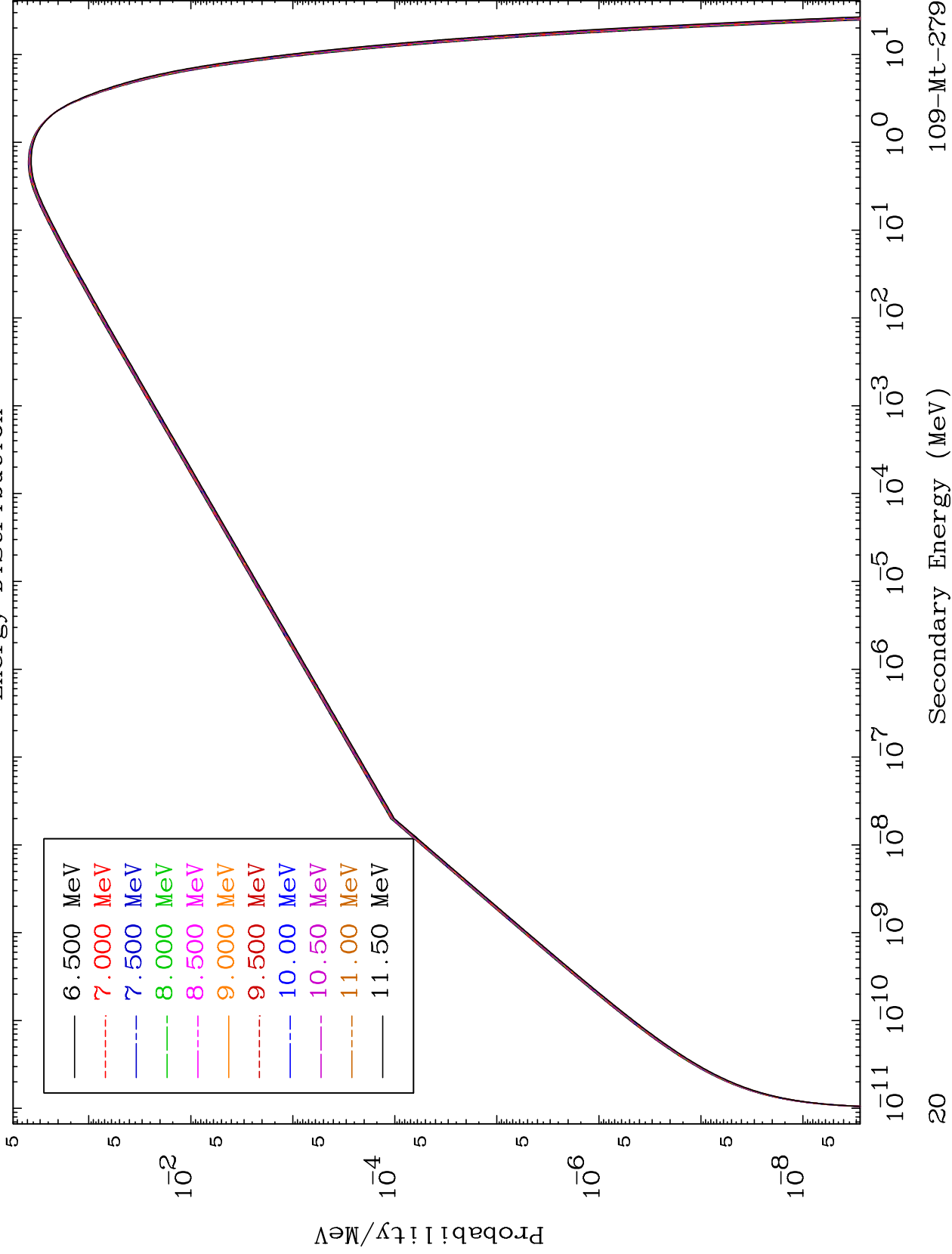
Secondary Energy (MeV)

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Fission Energy Distribution

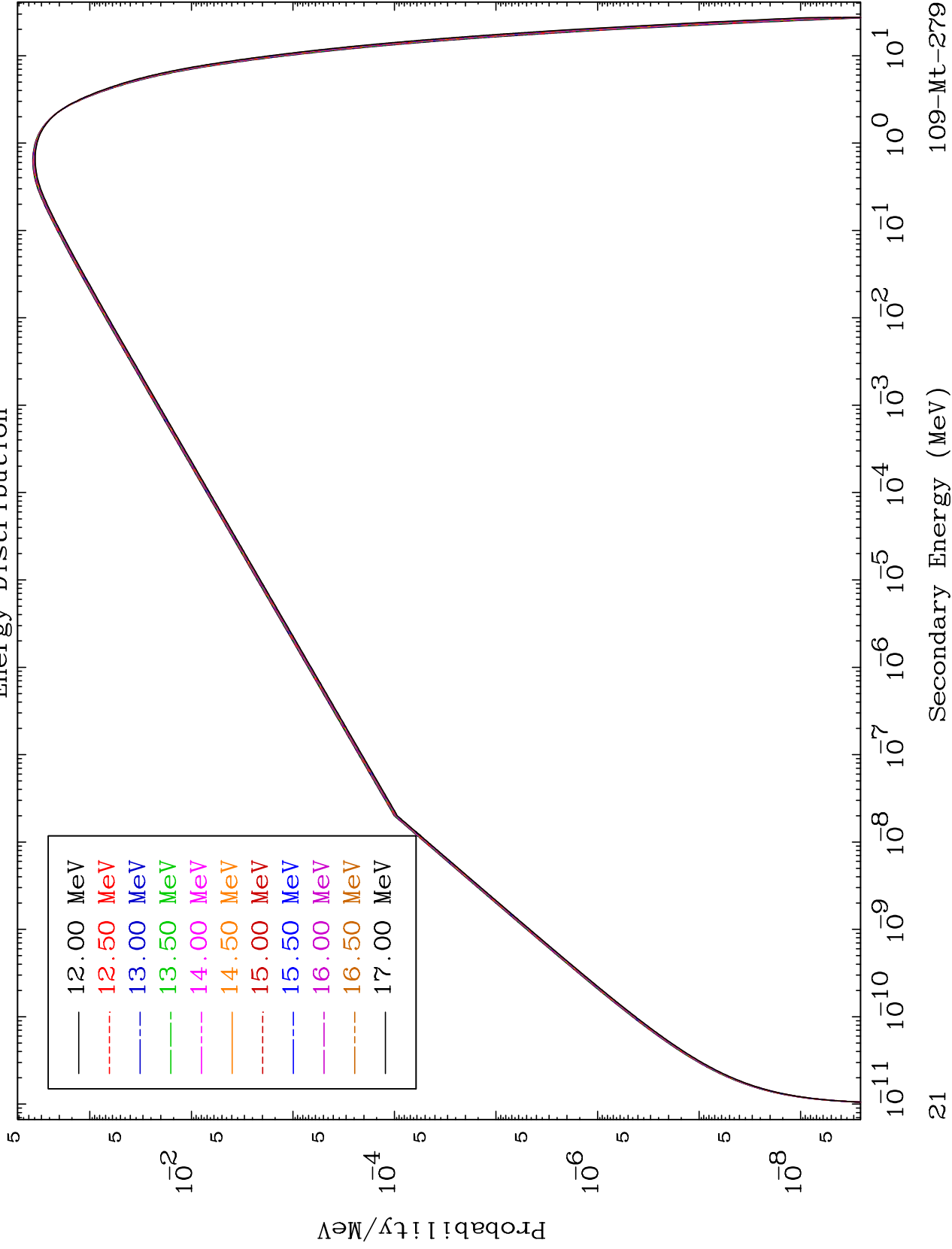
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Fission Energy Distribution

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Fission Energy Distribution

