

Program Complot
(Version 2018-1)

by

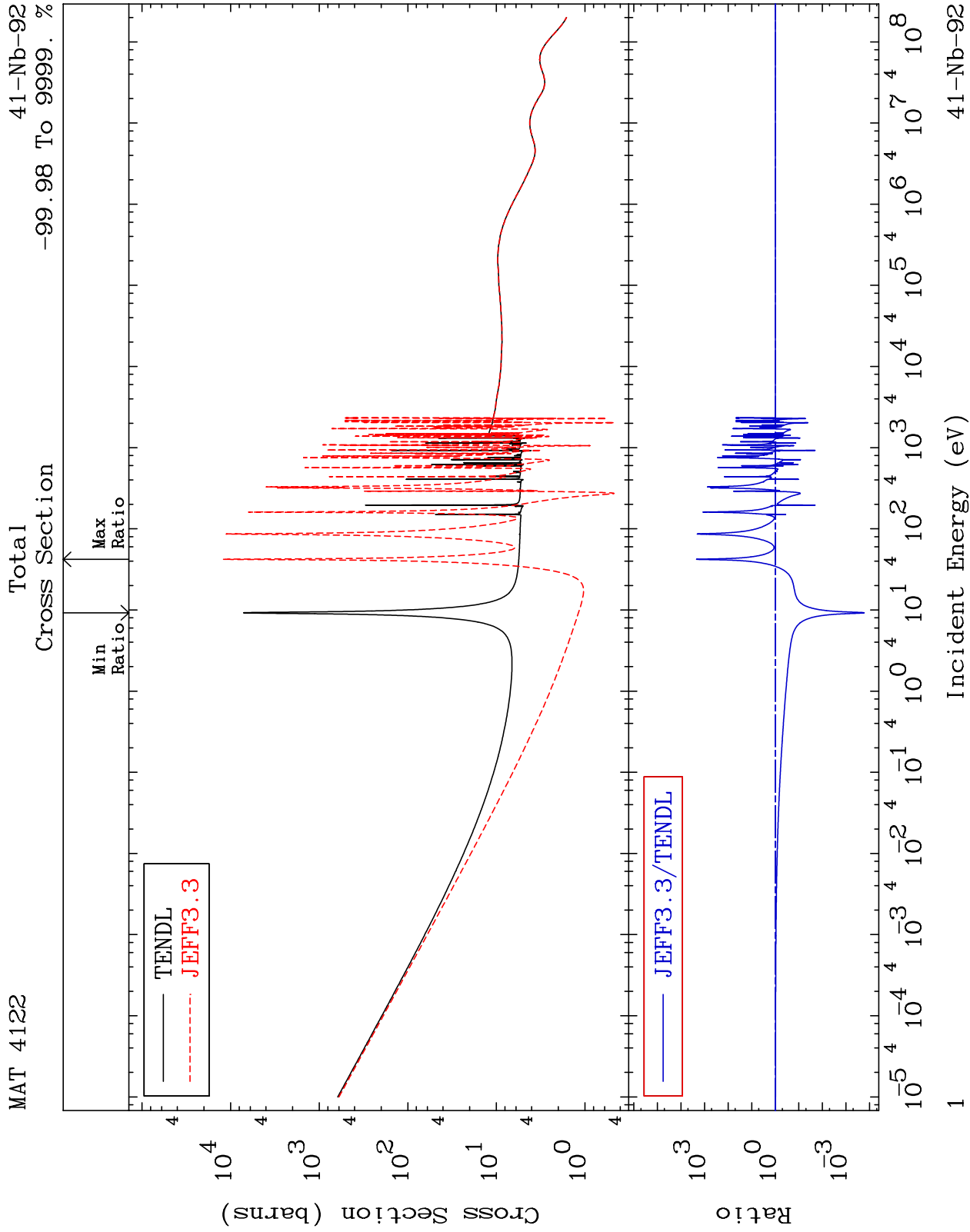
Dermott E. Cullen
(Present Contact Information)

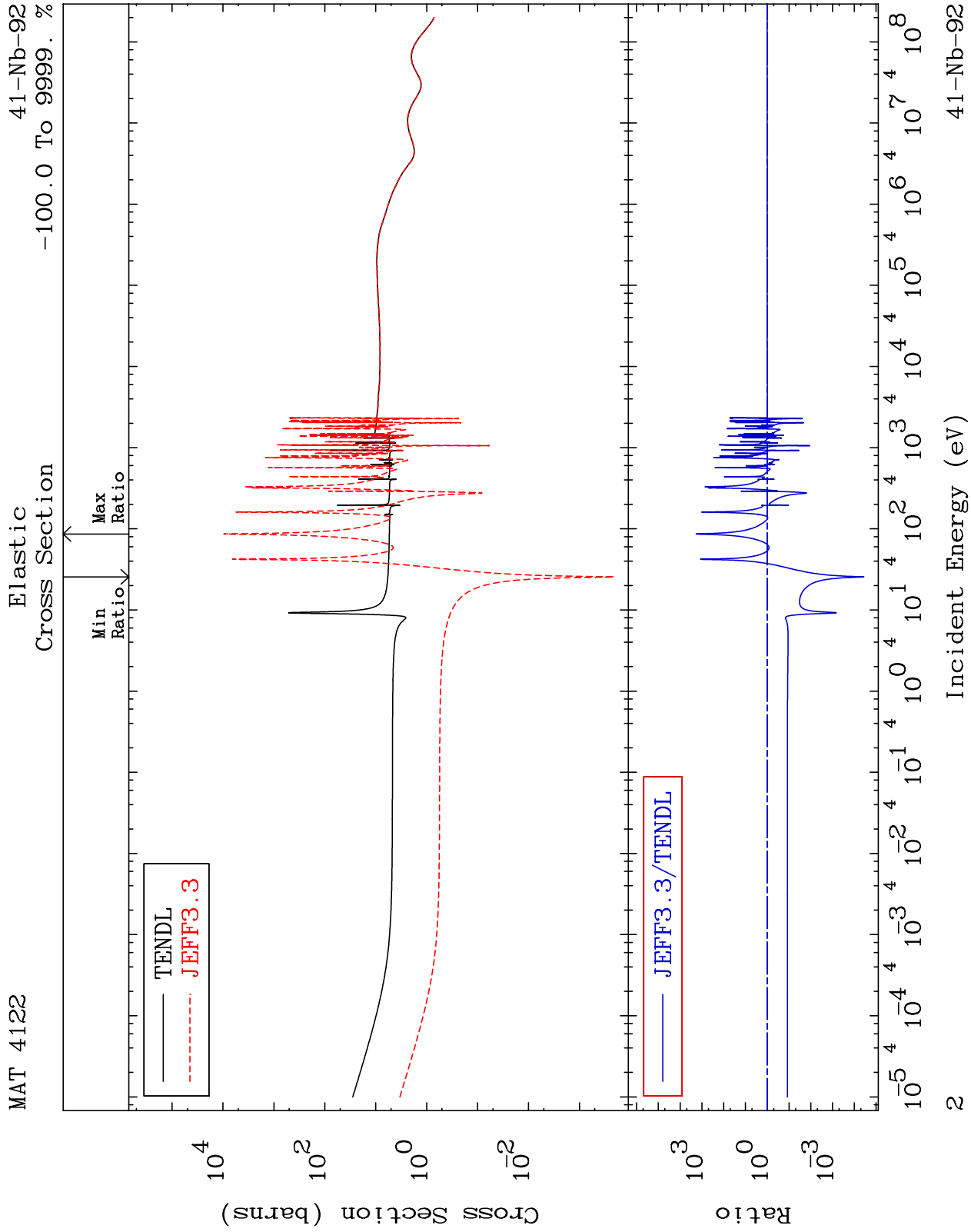
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

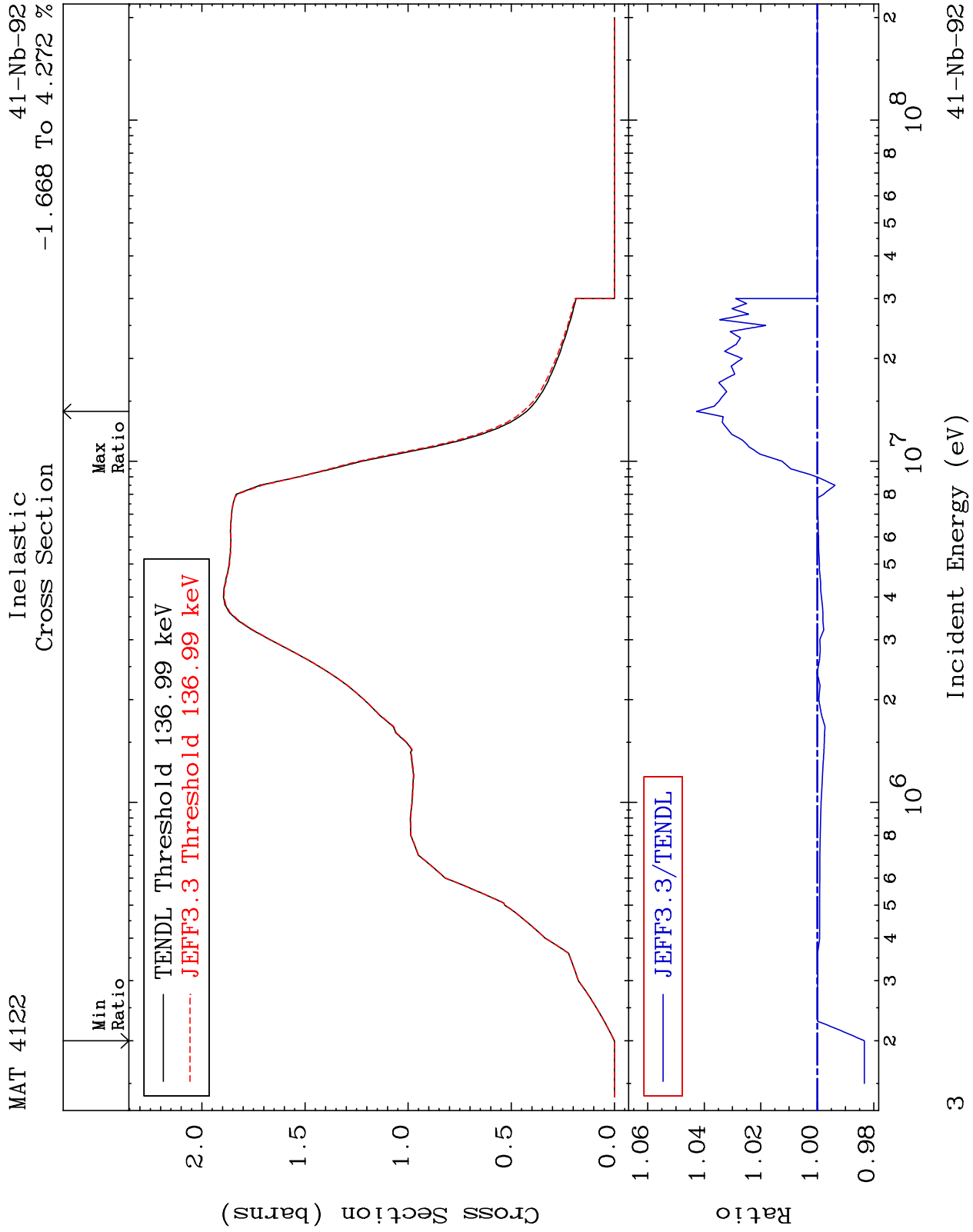
Tele: 925-443-1911

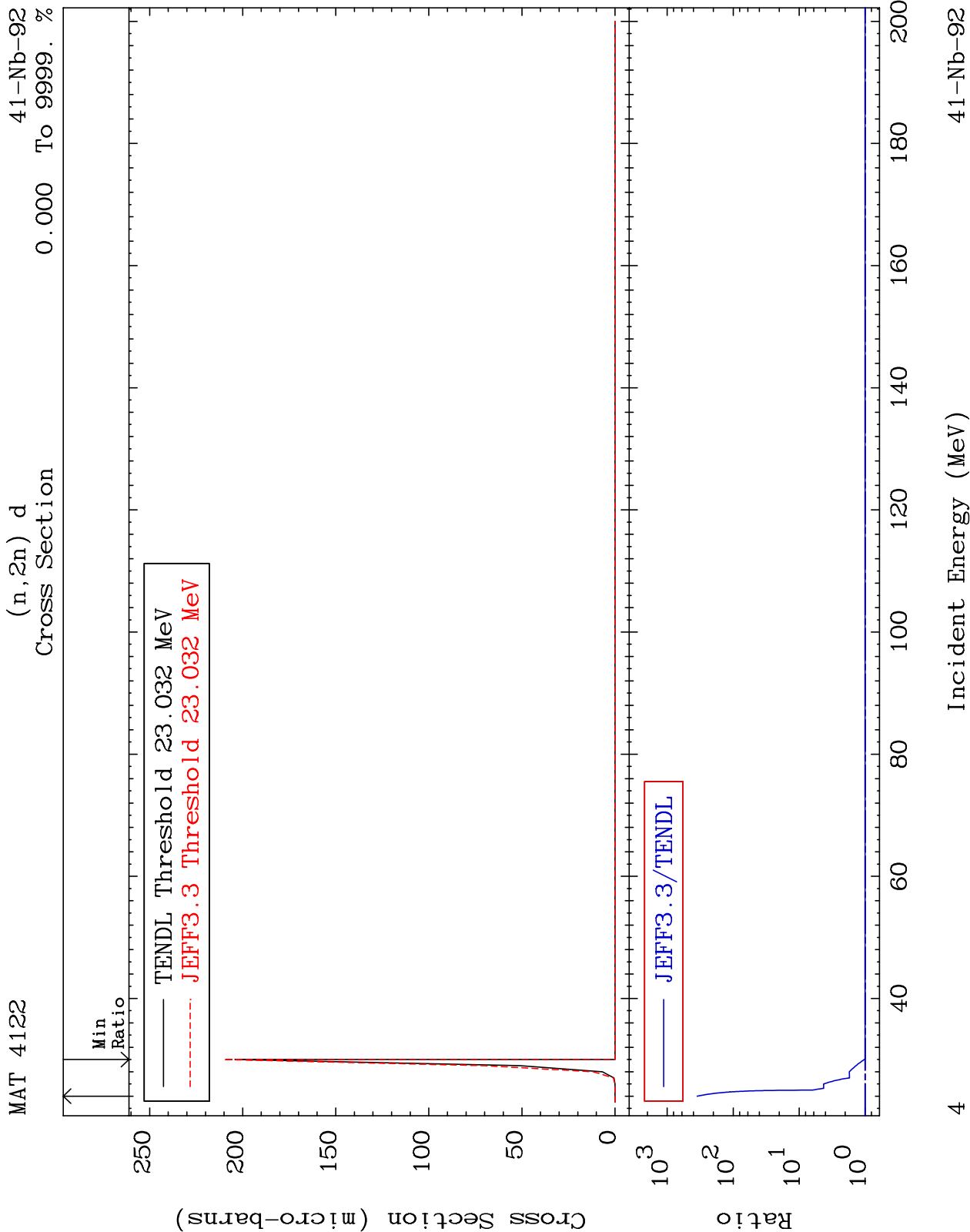
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

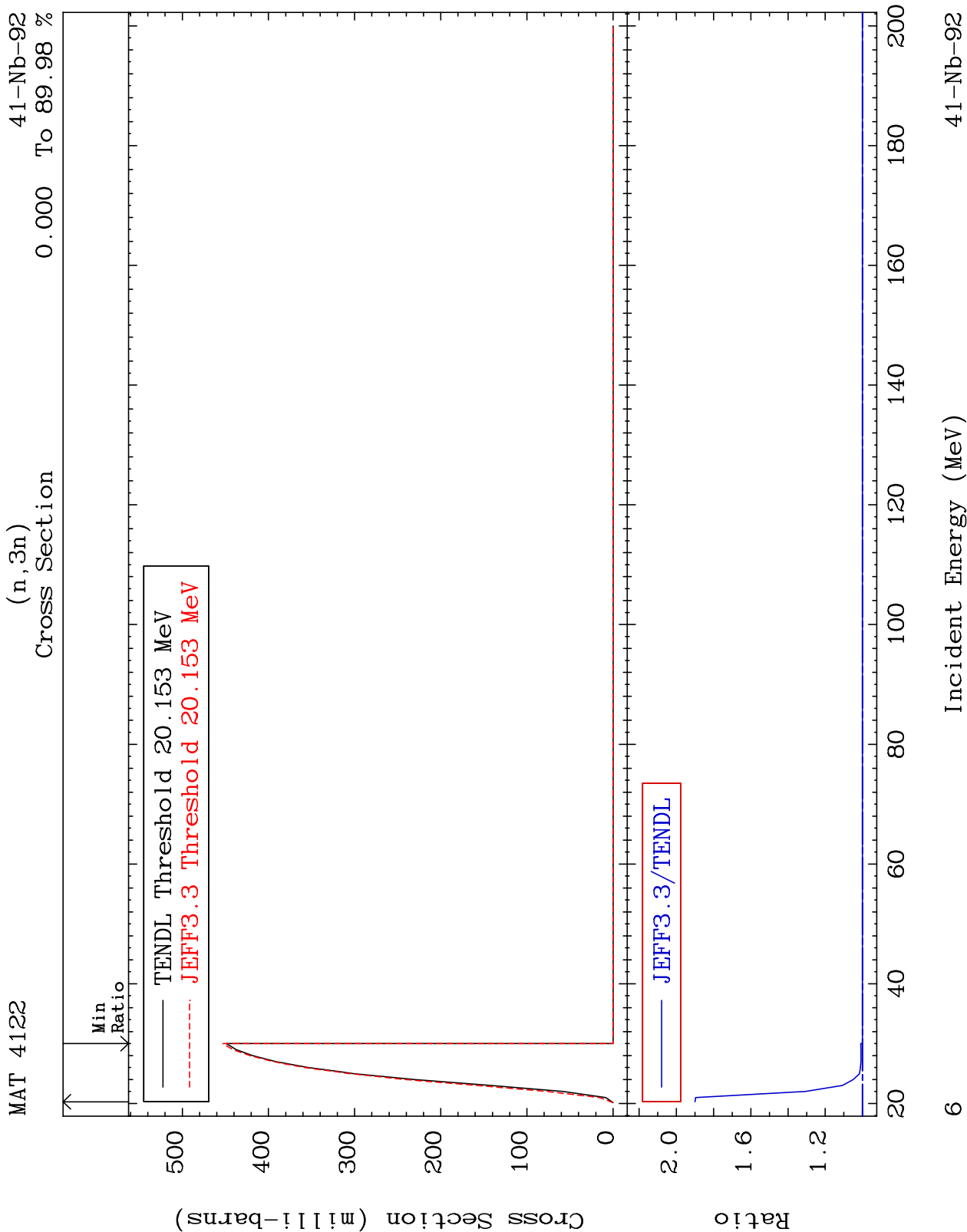
Press Mouse Button to Start

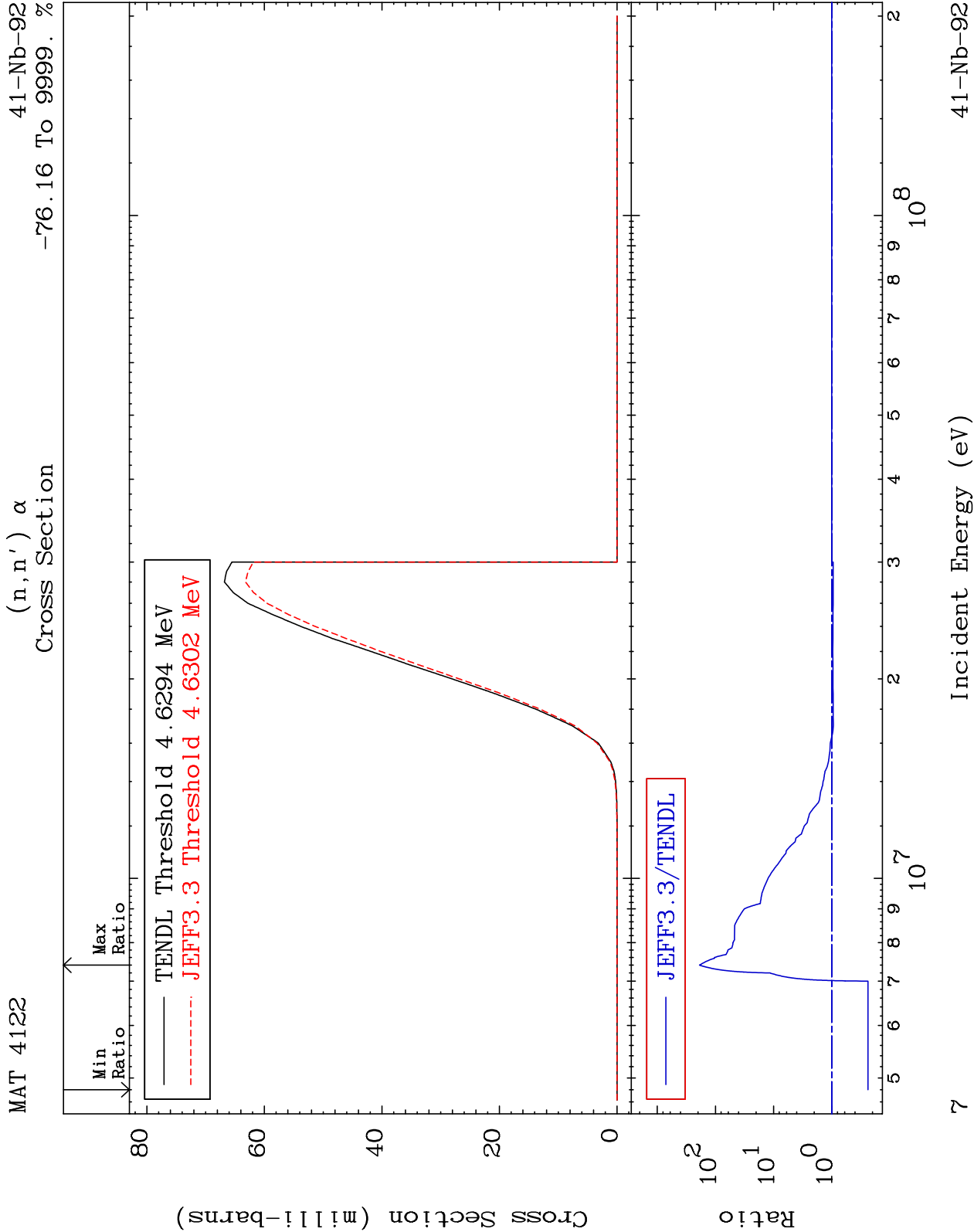


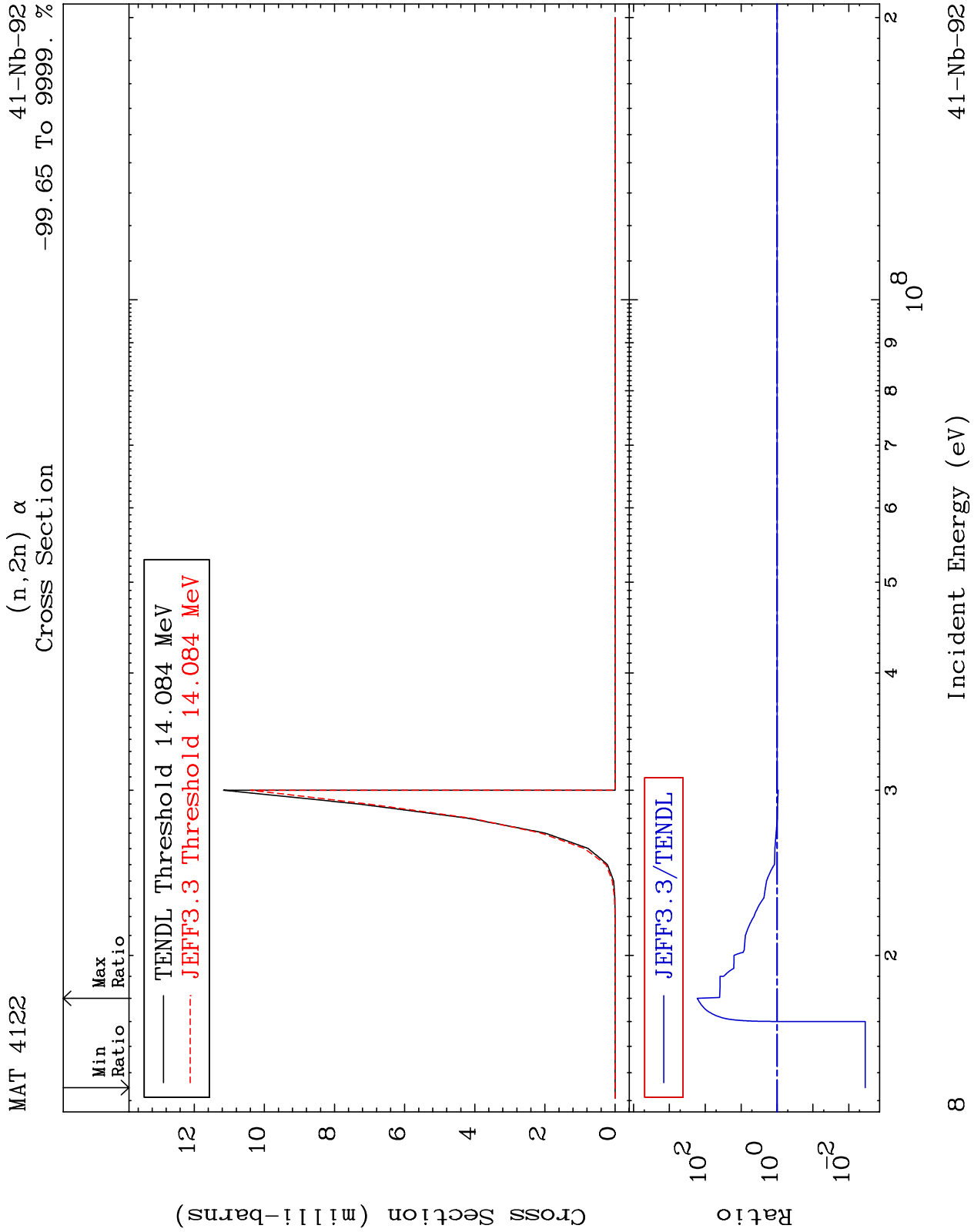












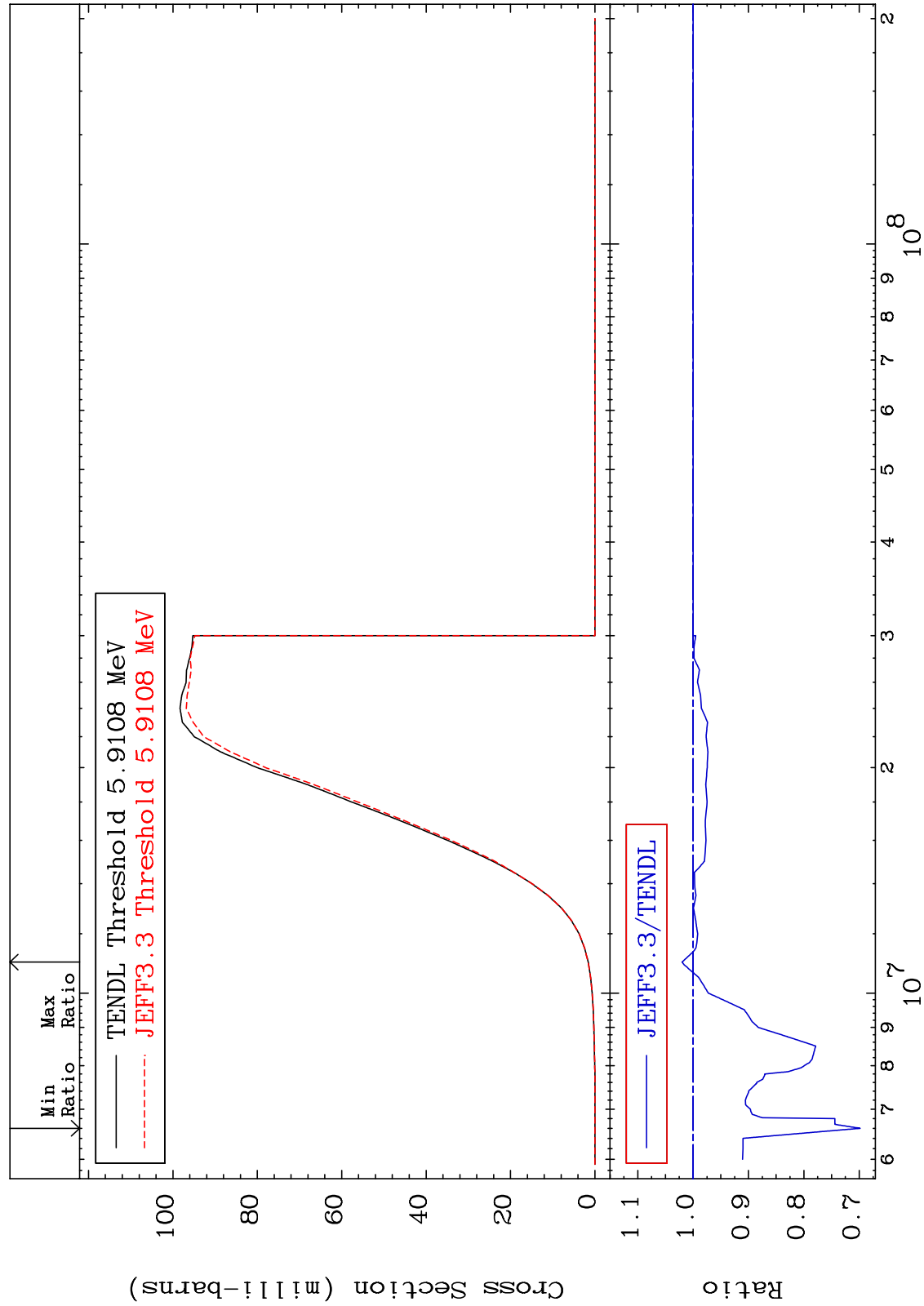
MAT 4122

(n,n') p

41-Nb-92

Cross Section

-30.17 To 1.994 %



9

Incident Energy (eV)

41-Nb-92

MAT 4122

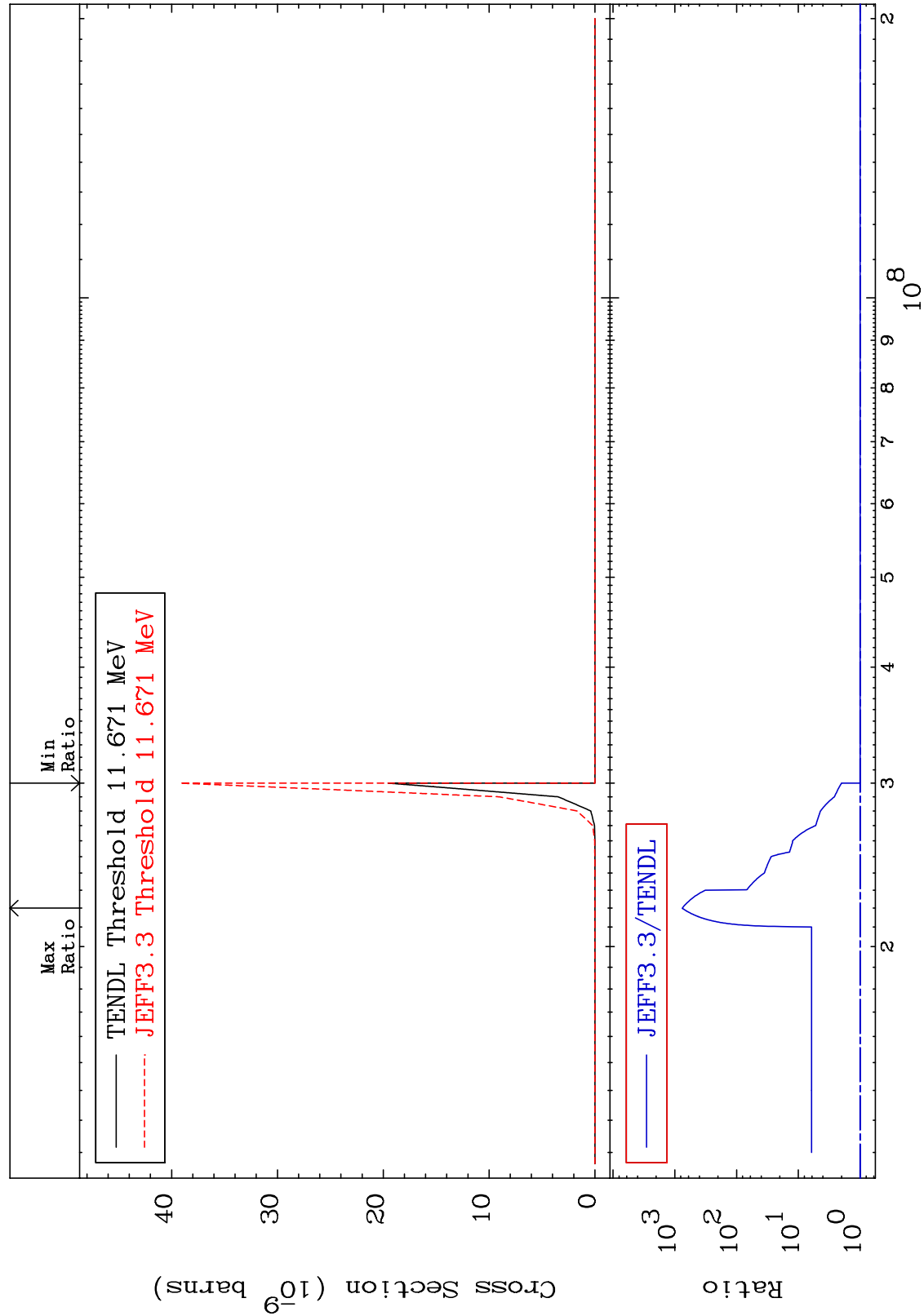
(n,n') 2α

41-Nb-92

Cross Section

0.000

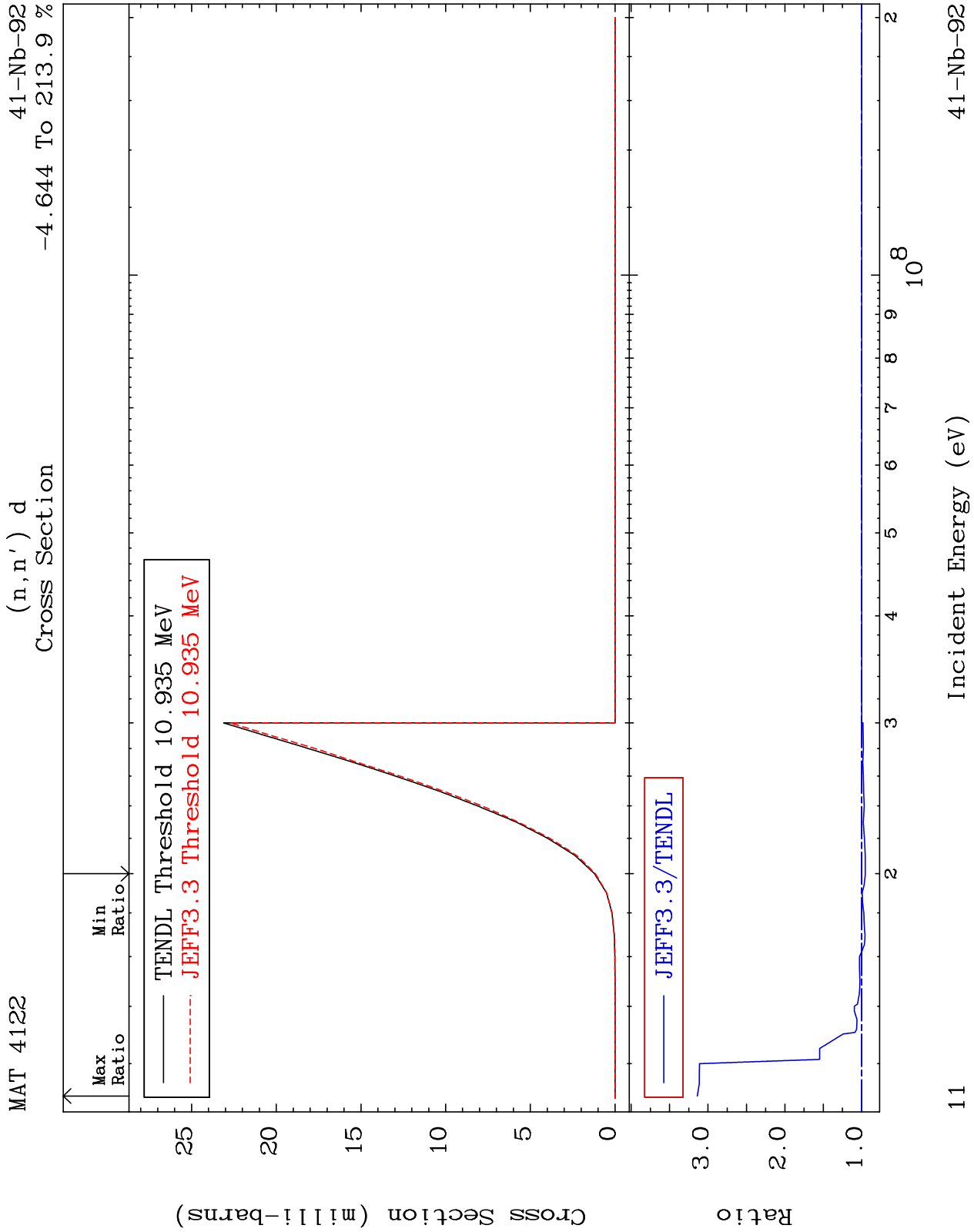
To 9999. %

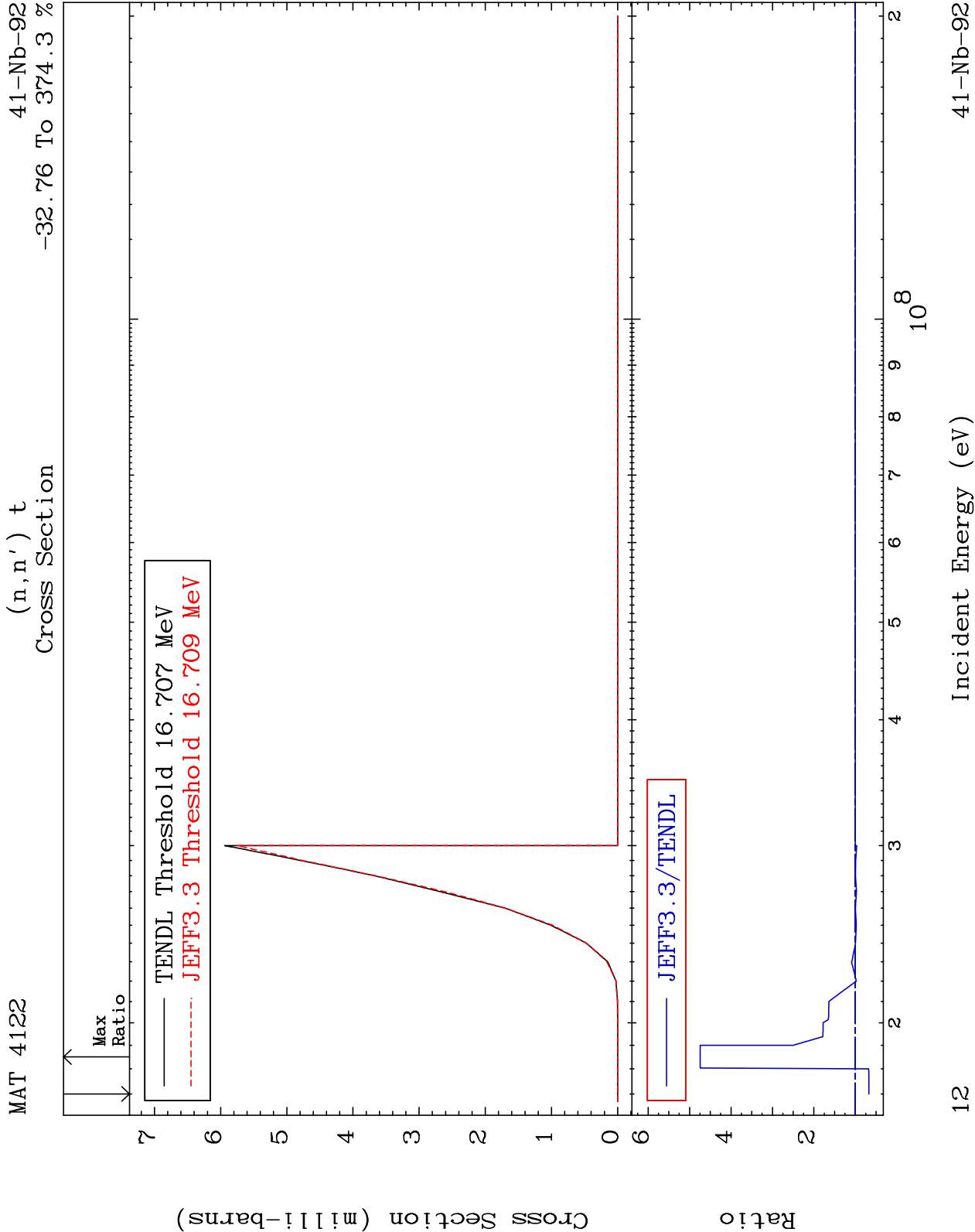


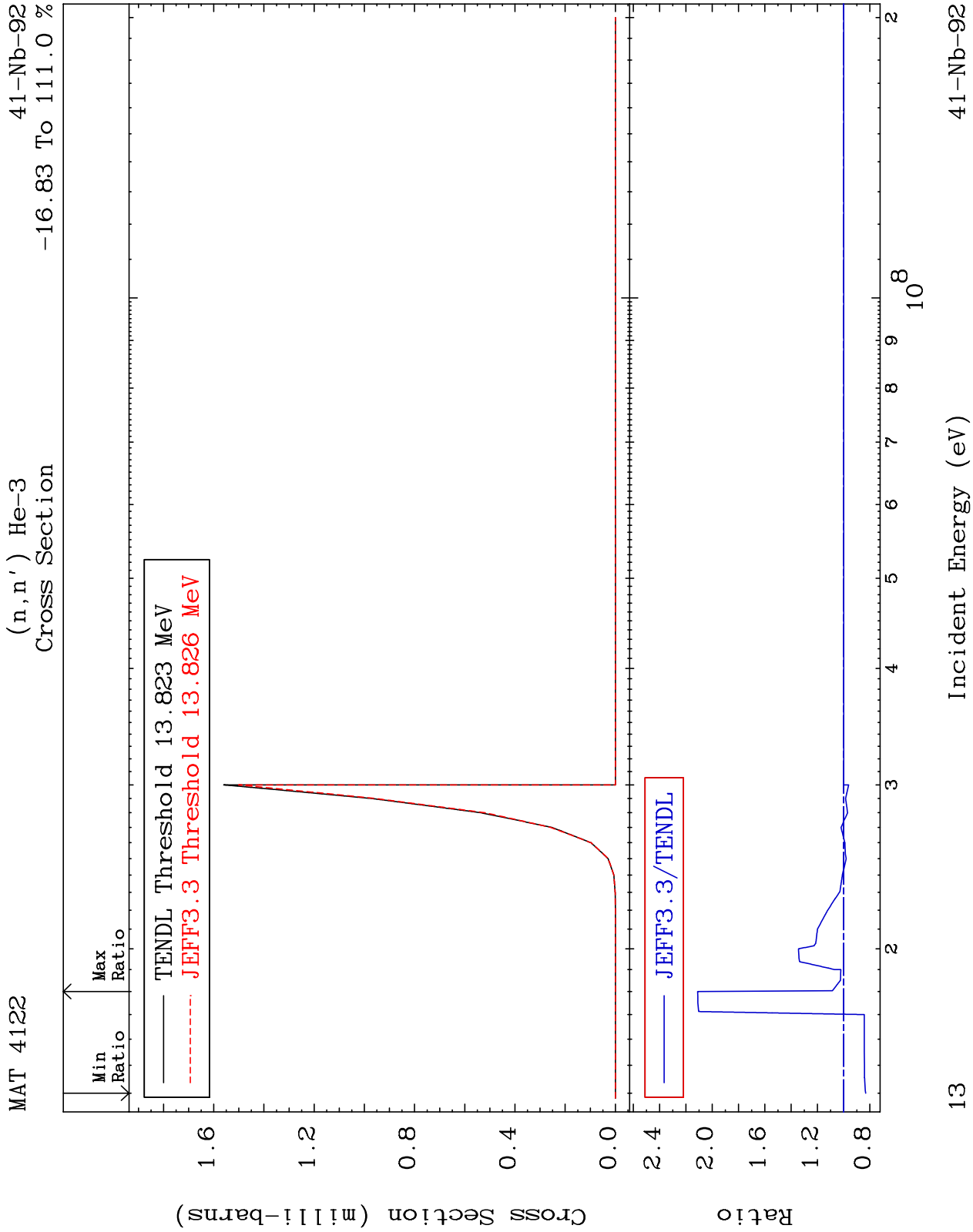
10

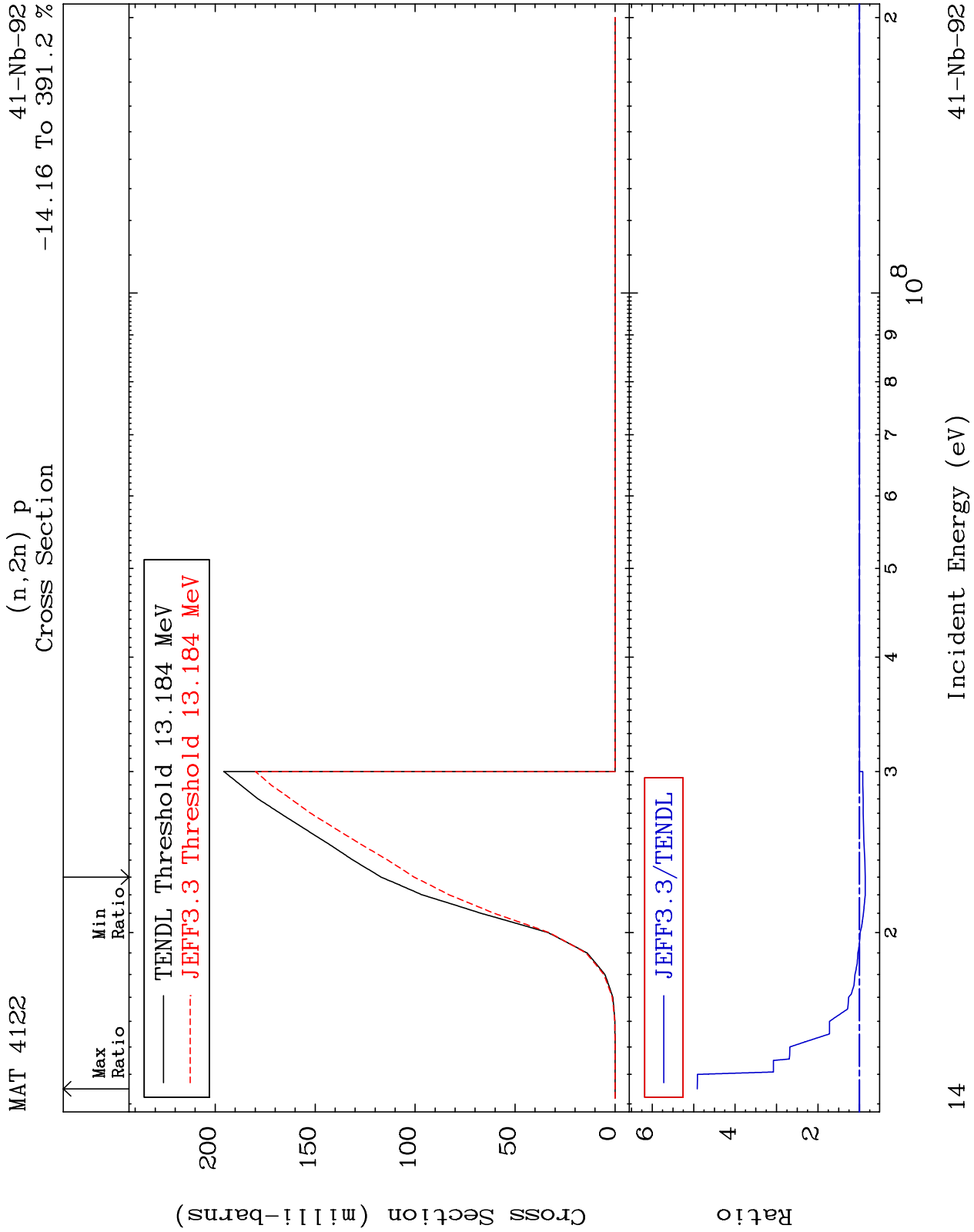
Incident Energy (eV)

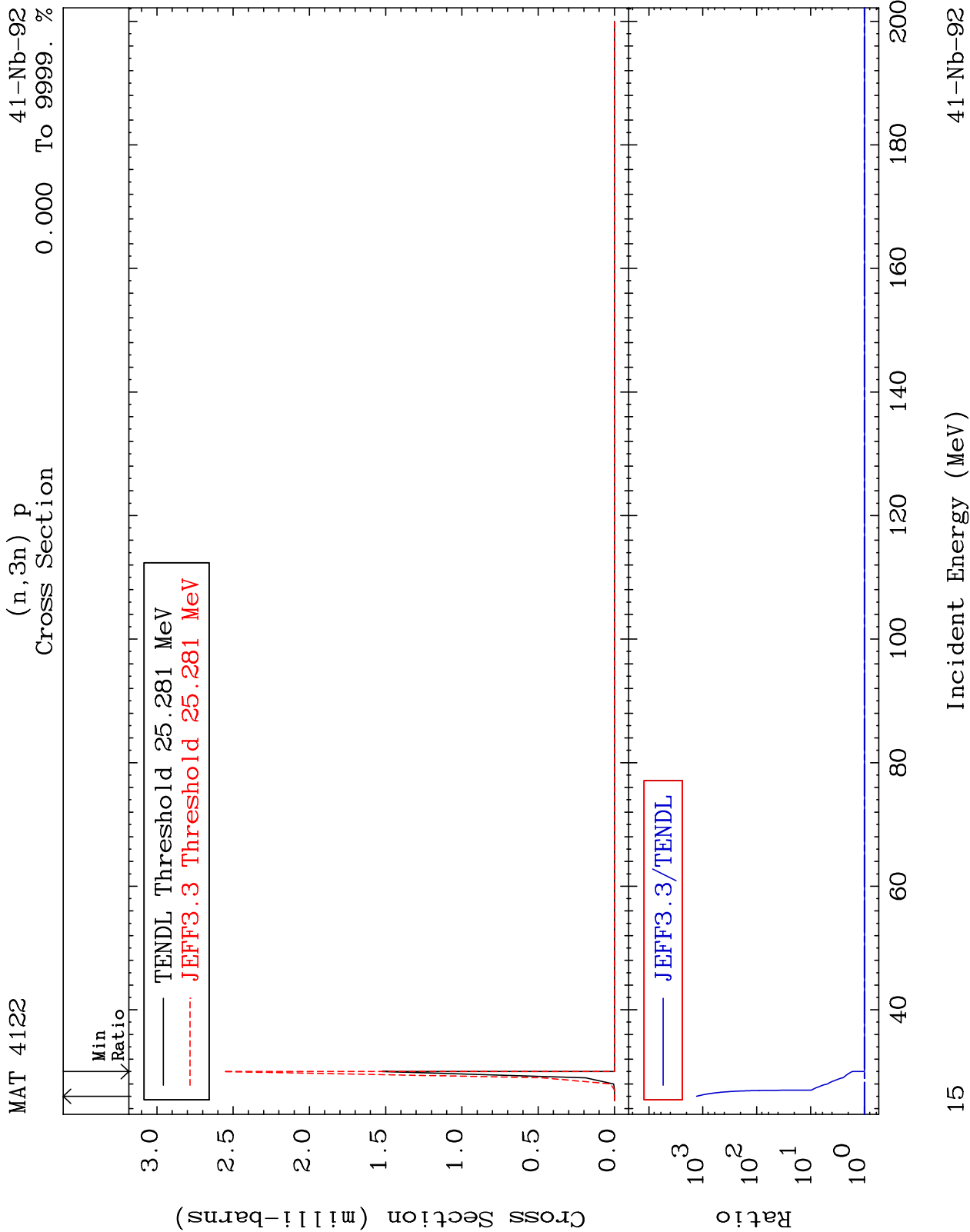
41-Nb-92

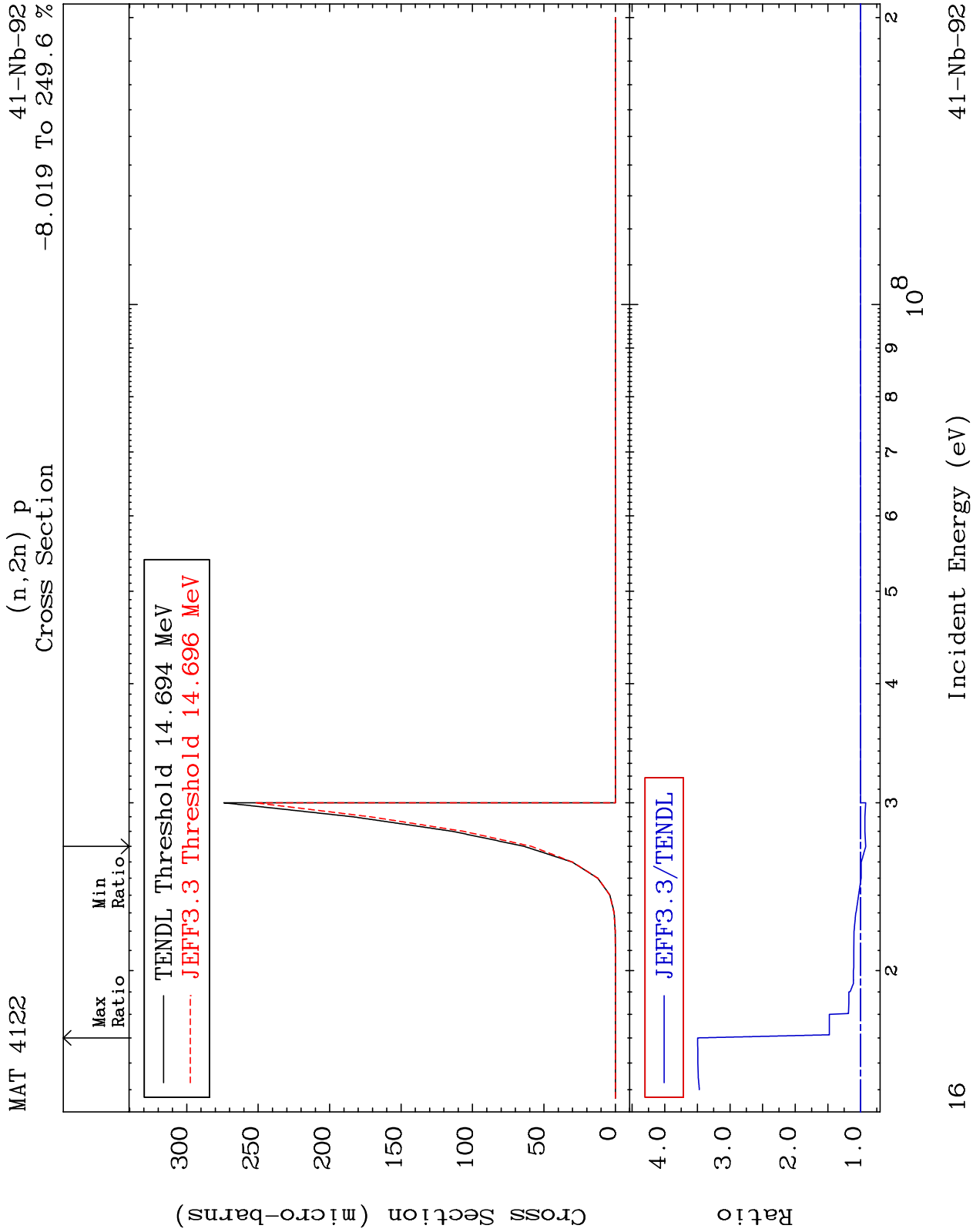


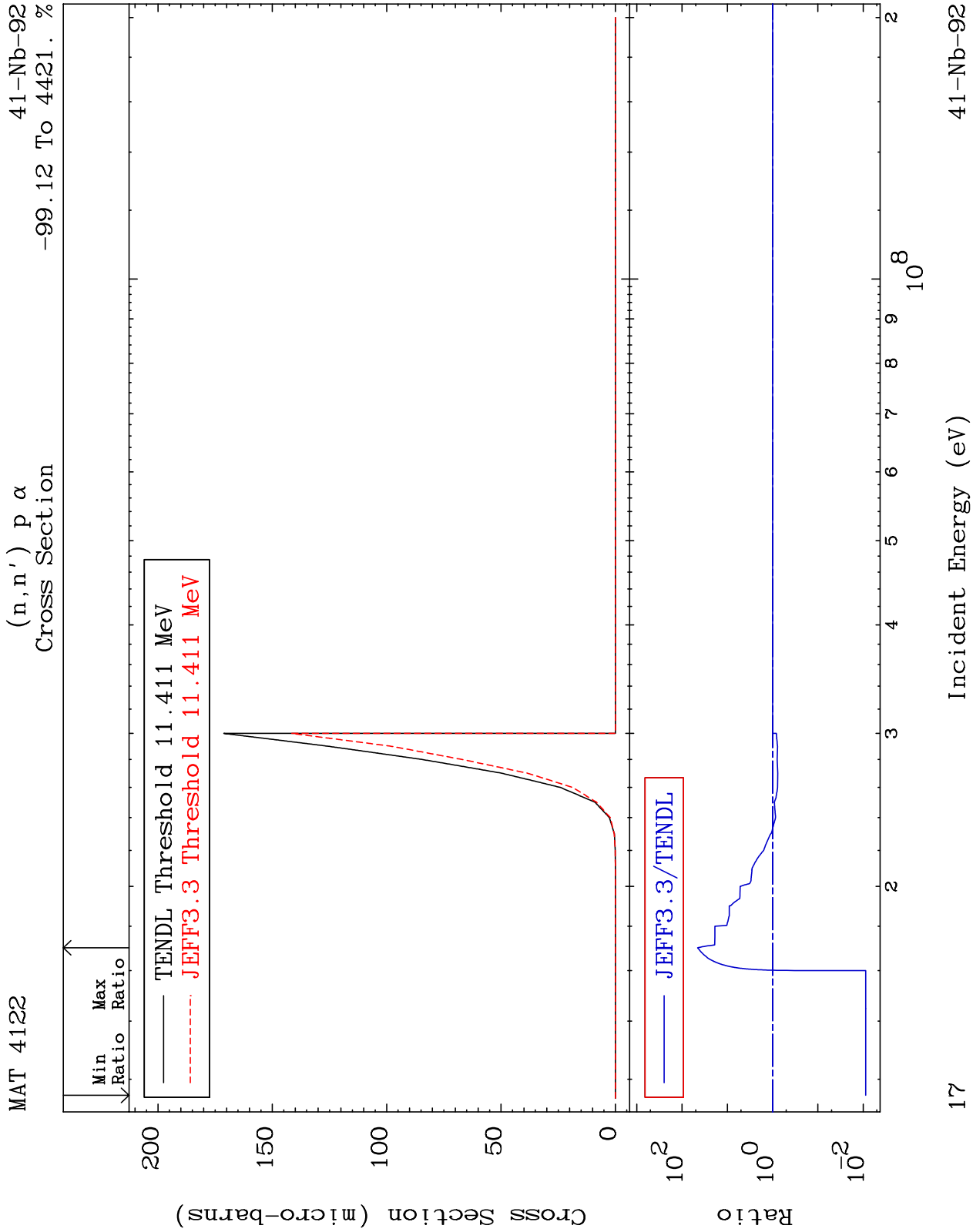


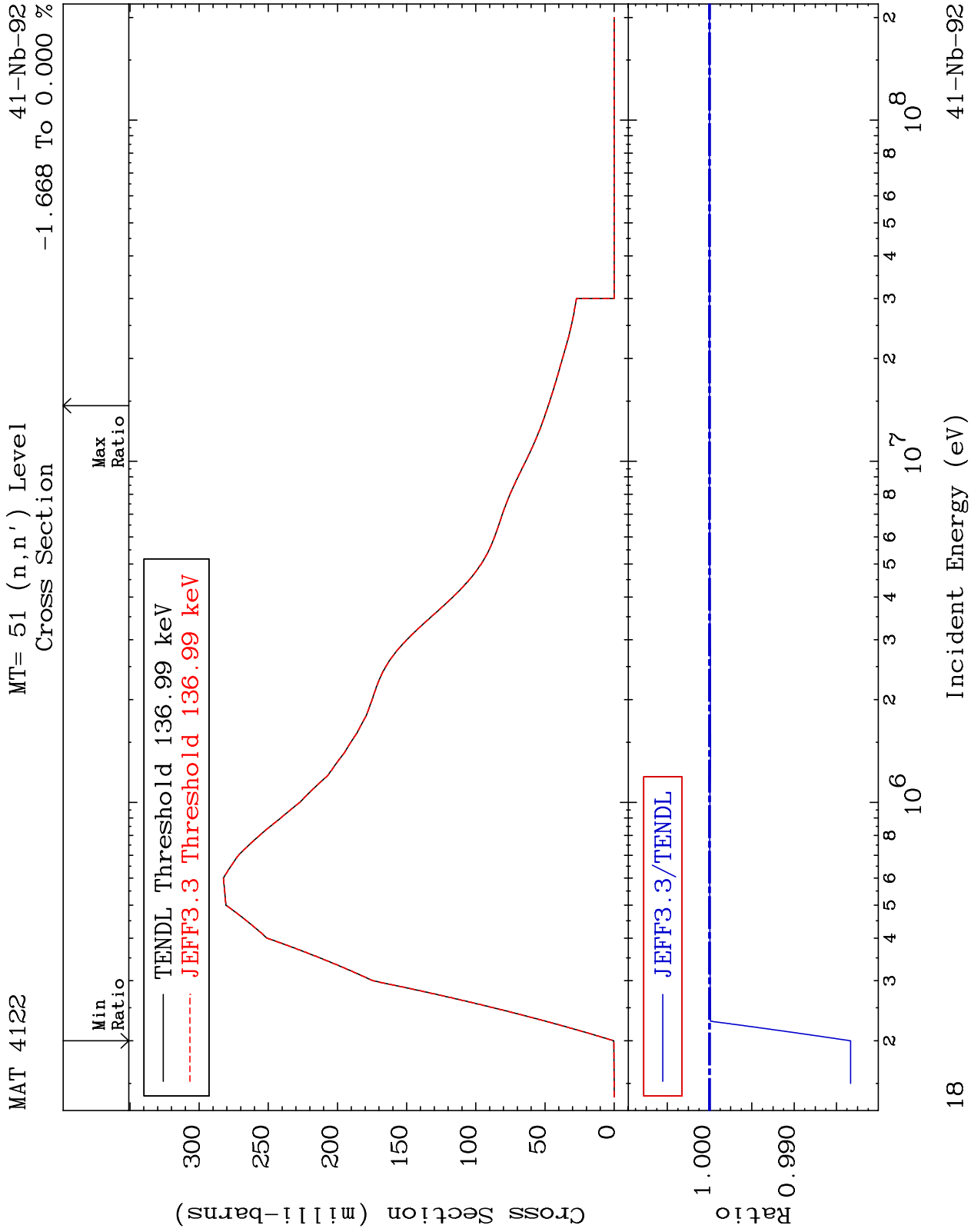


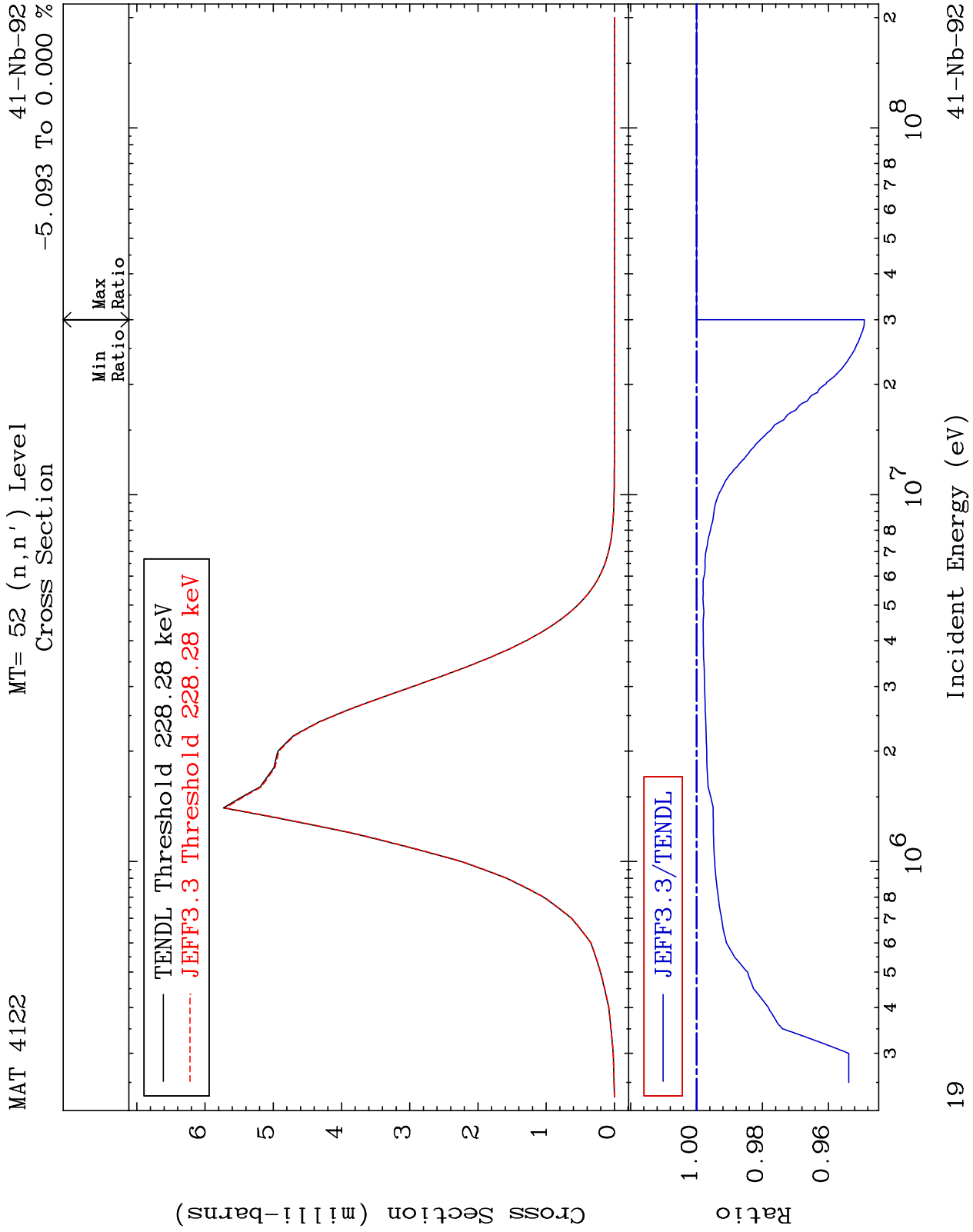


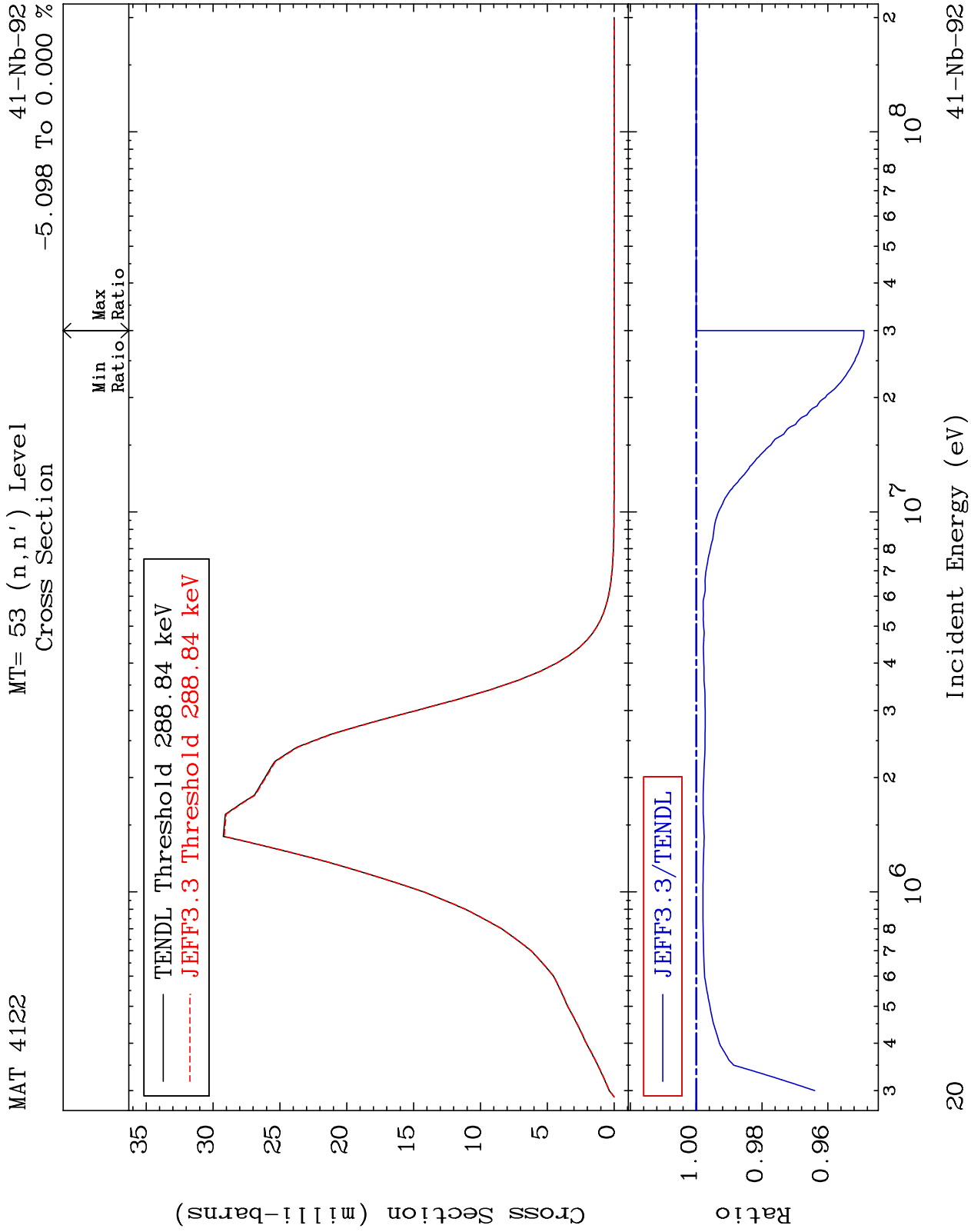


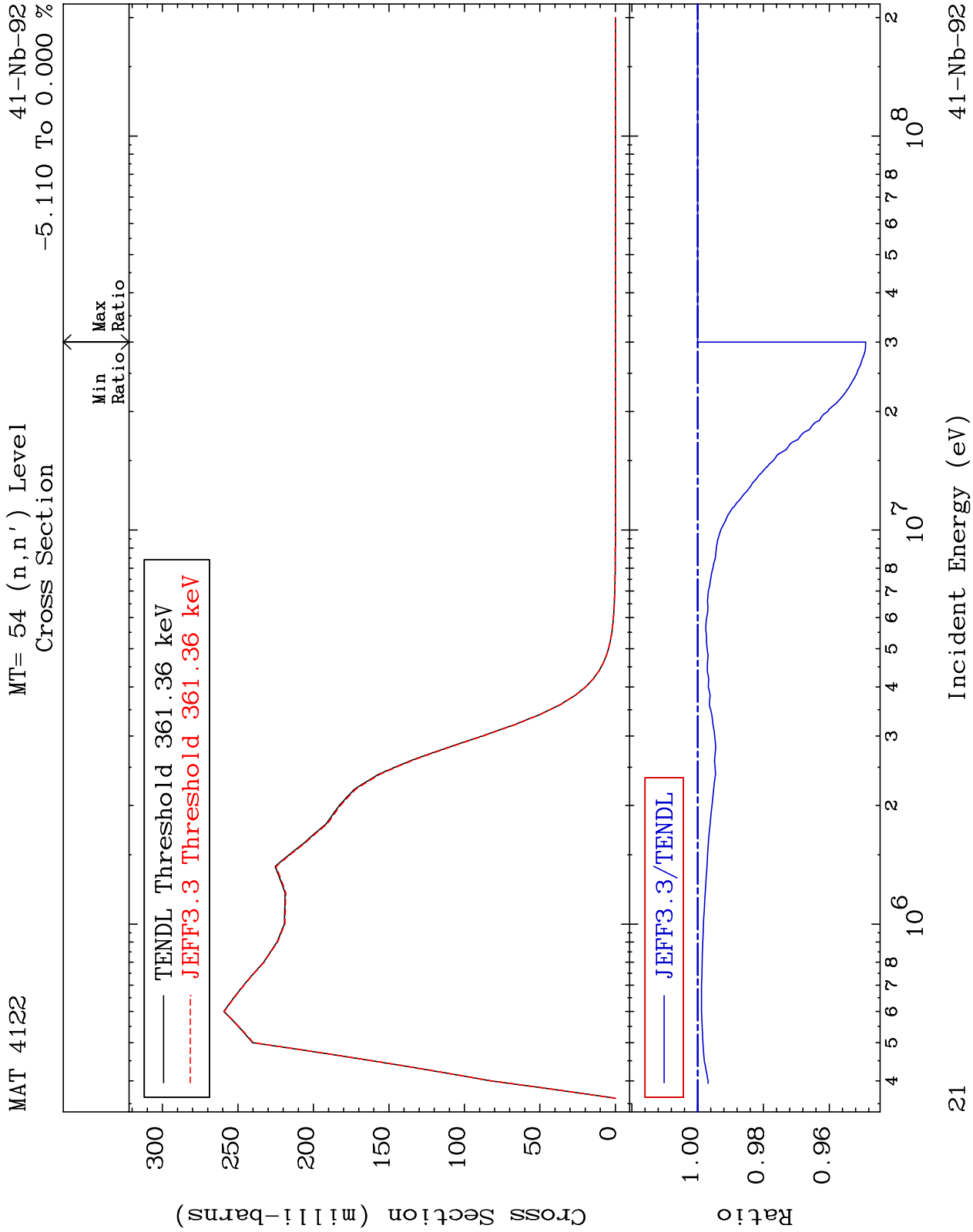


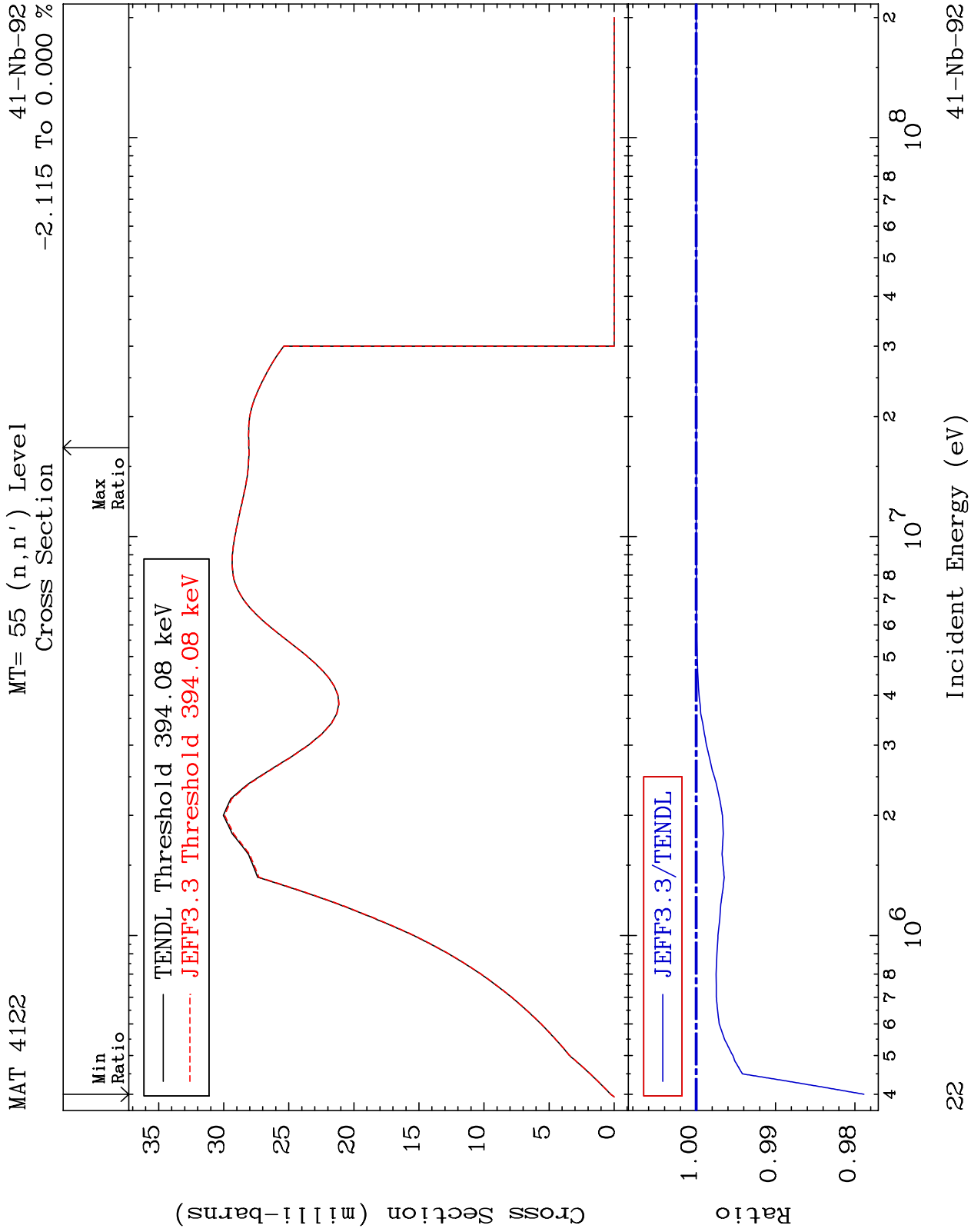


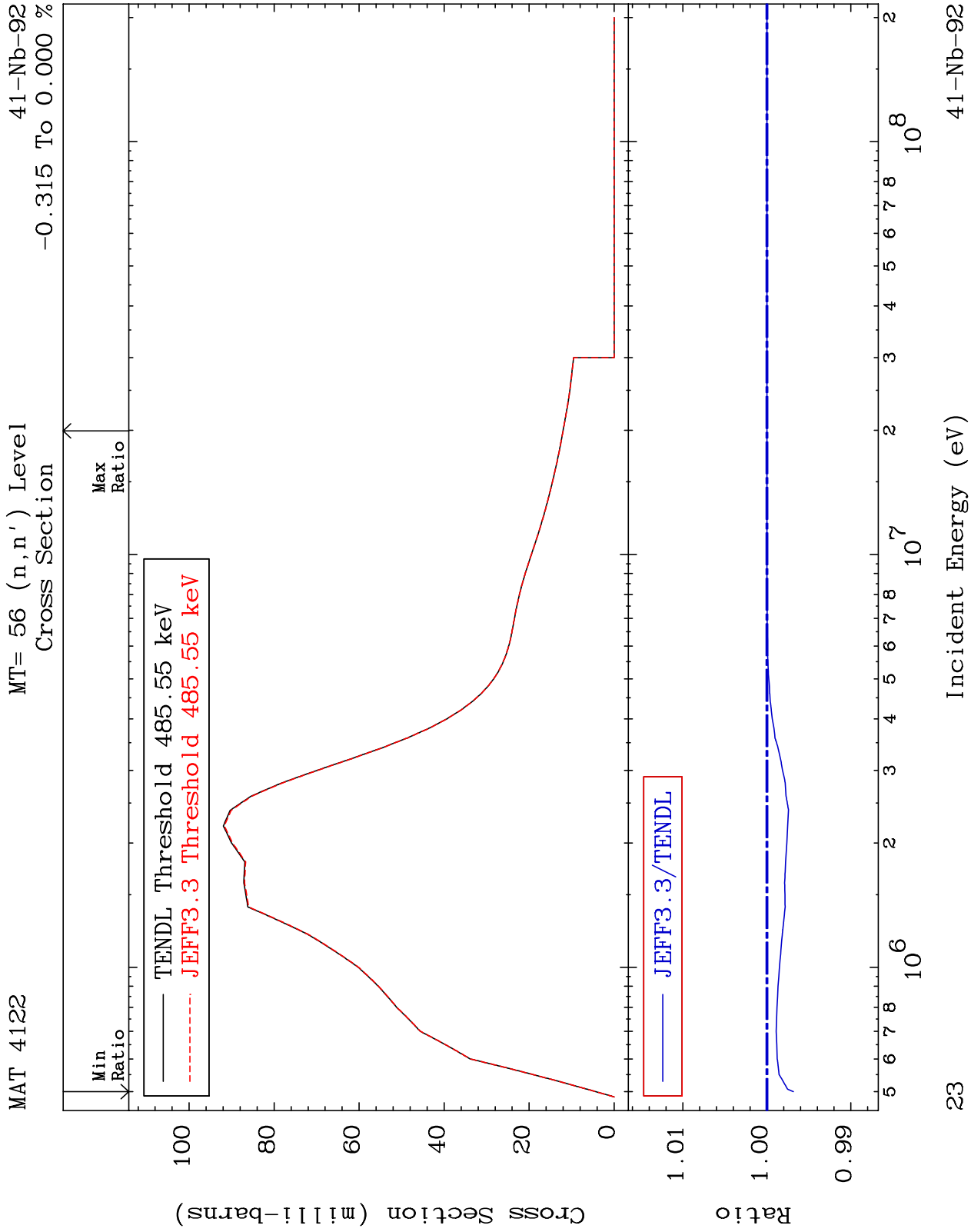


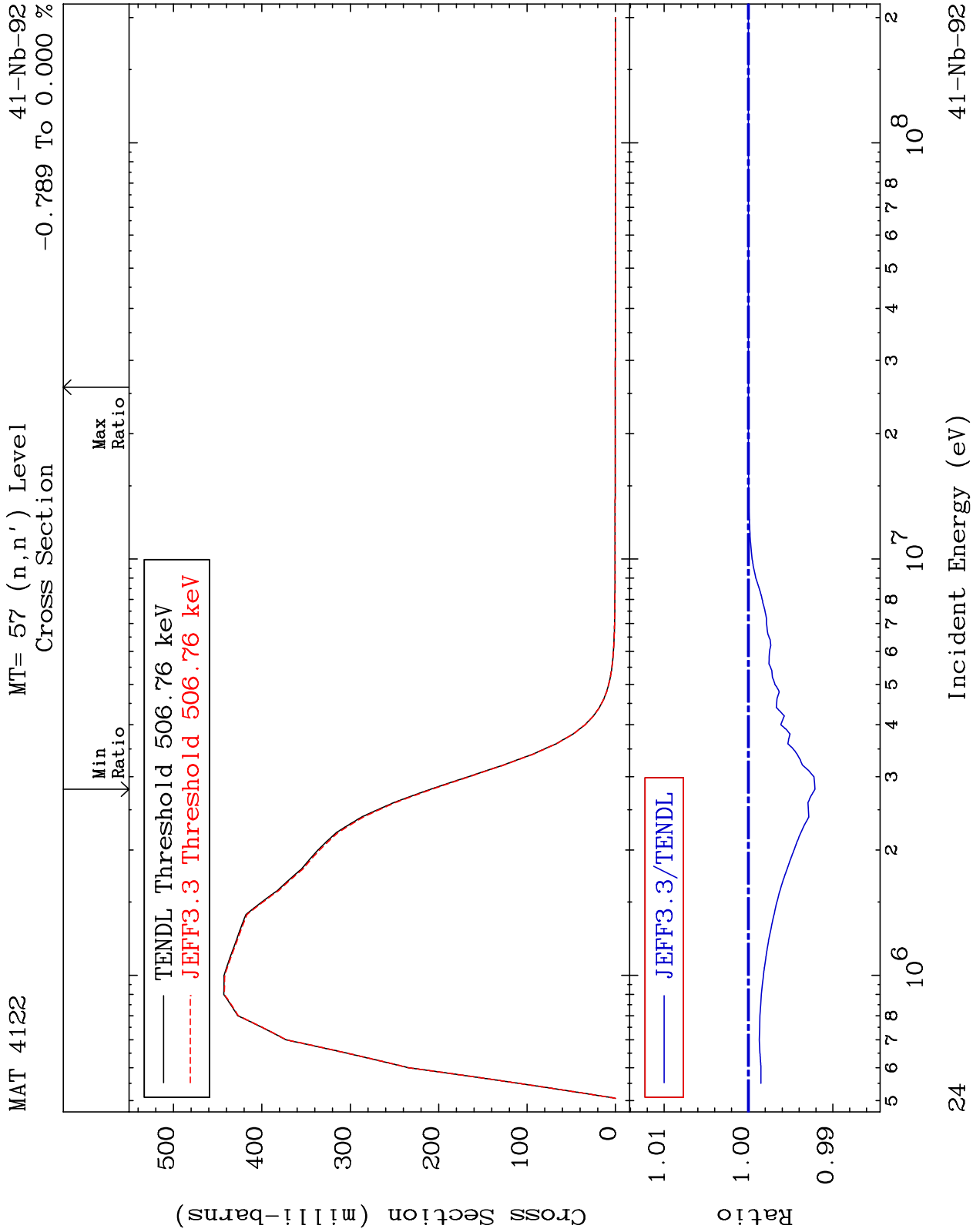












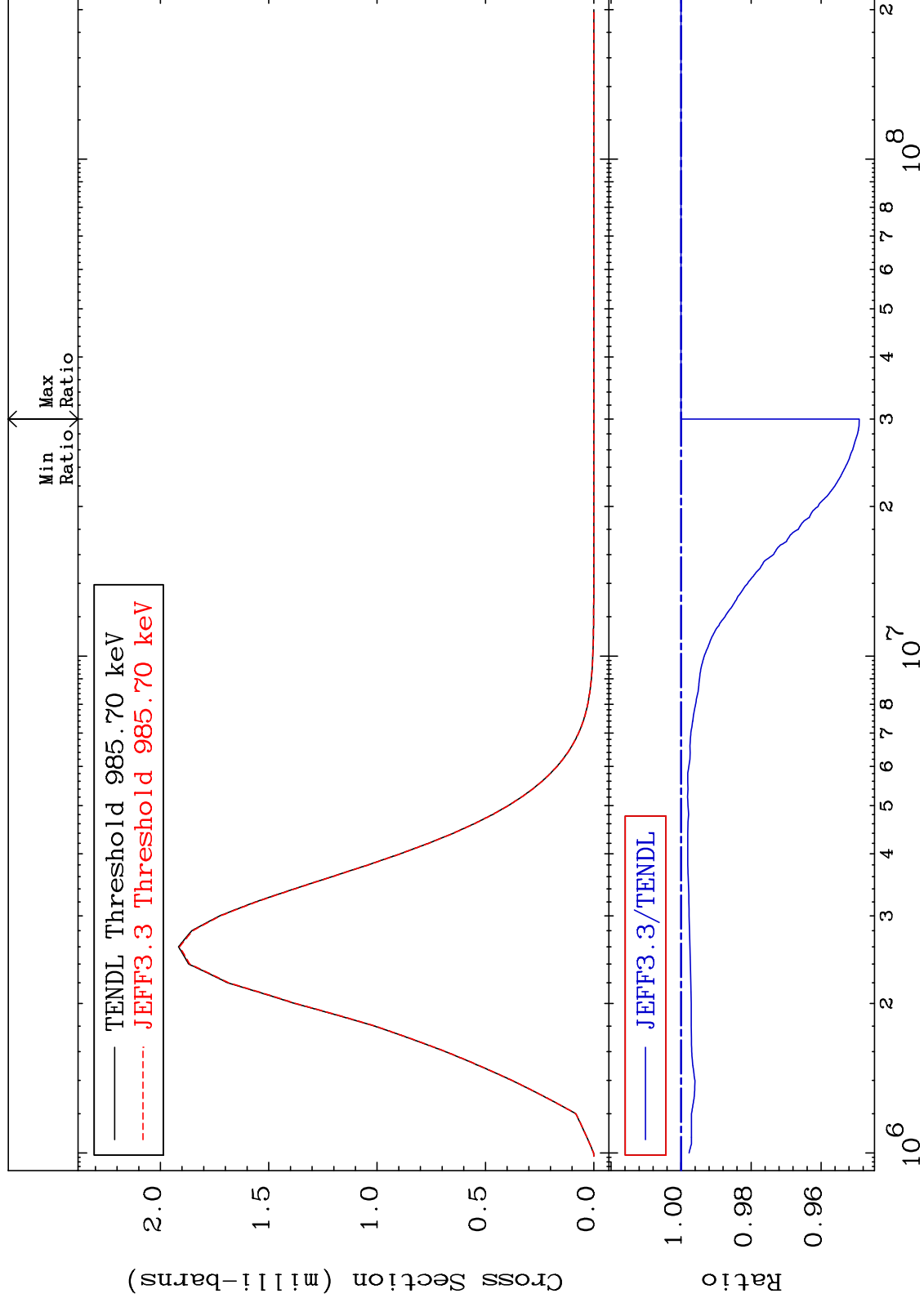
MAT 4122

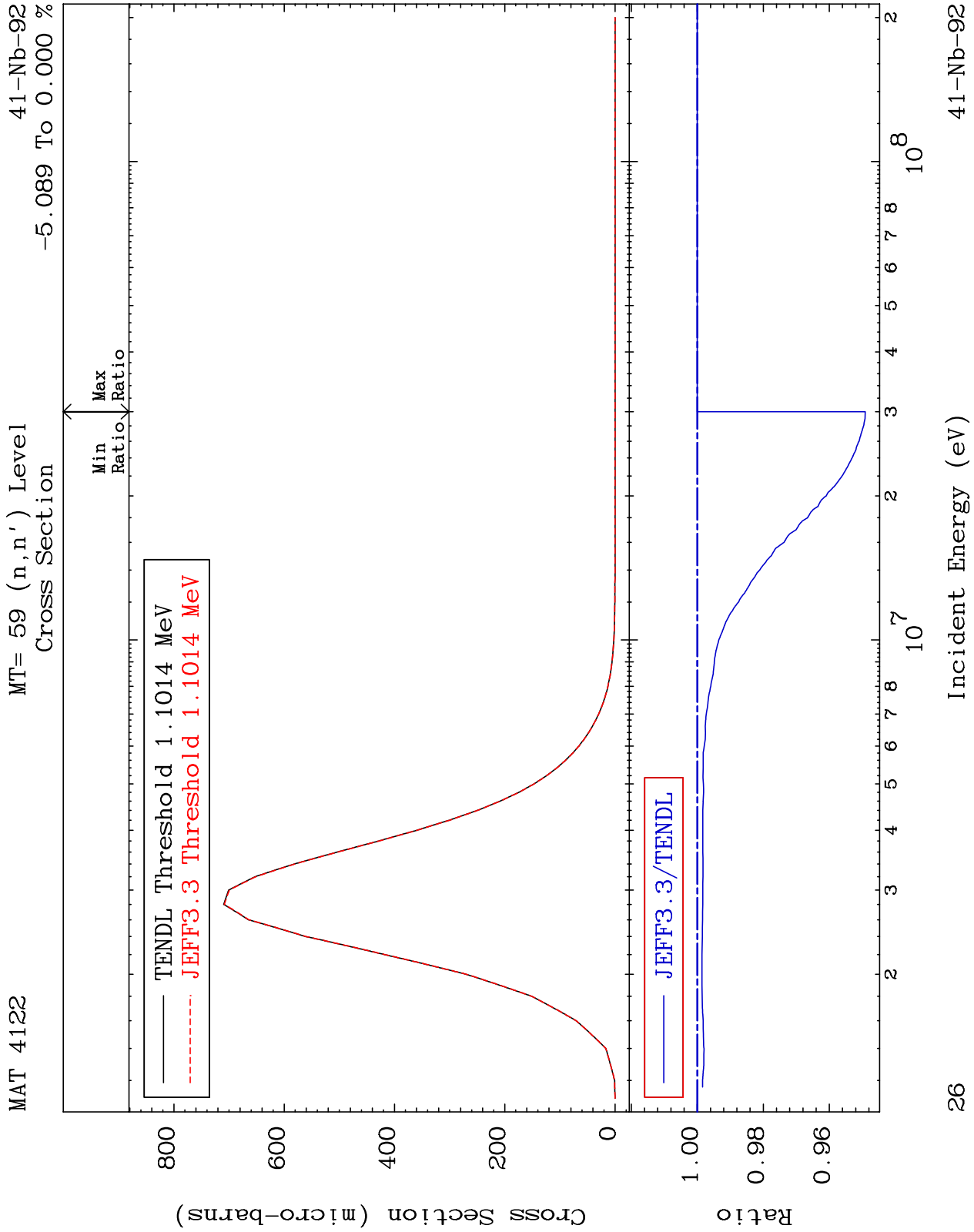
MT= 58 (n, n') Level

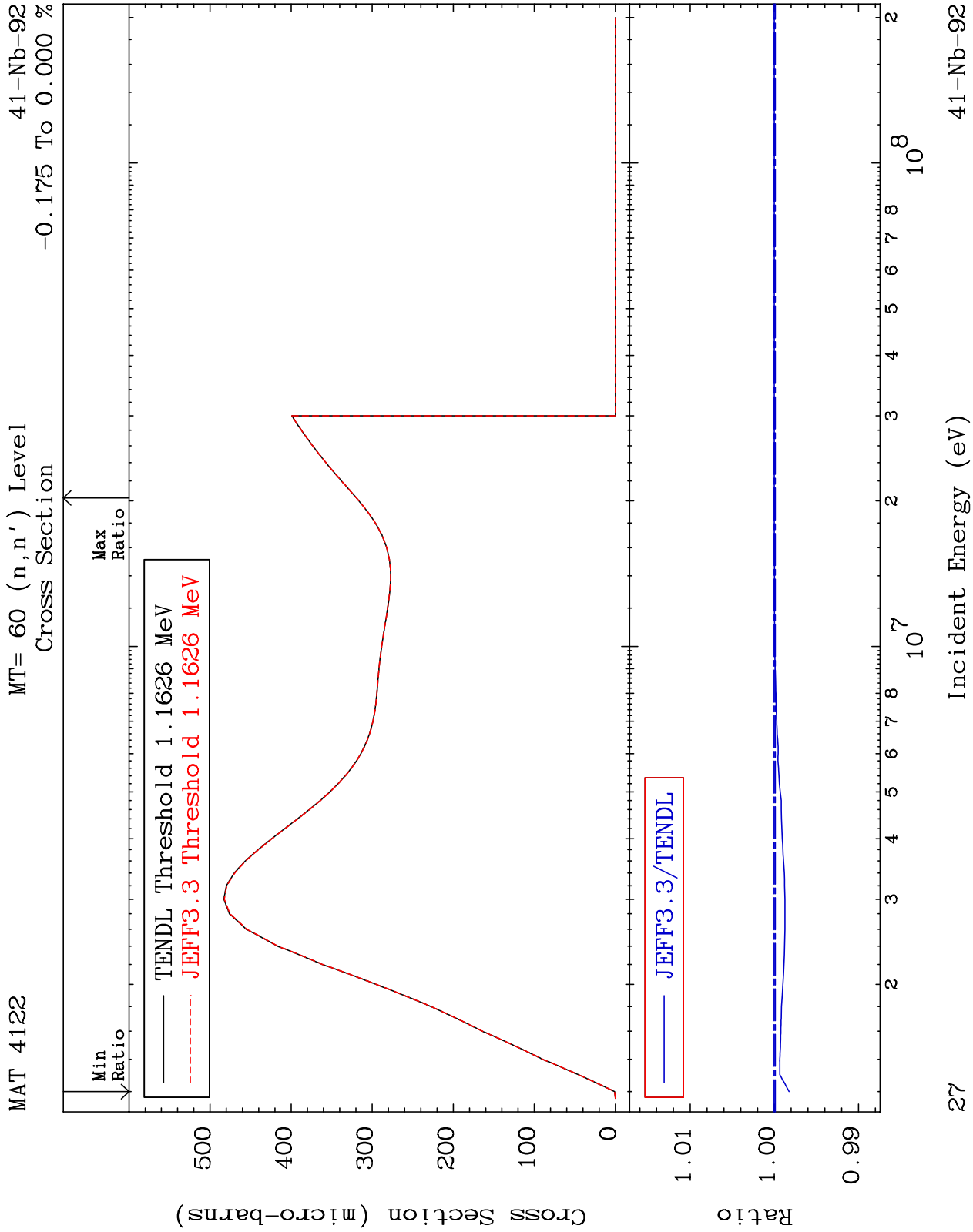
41-Nb-92

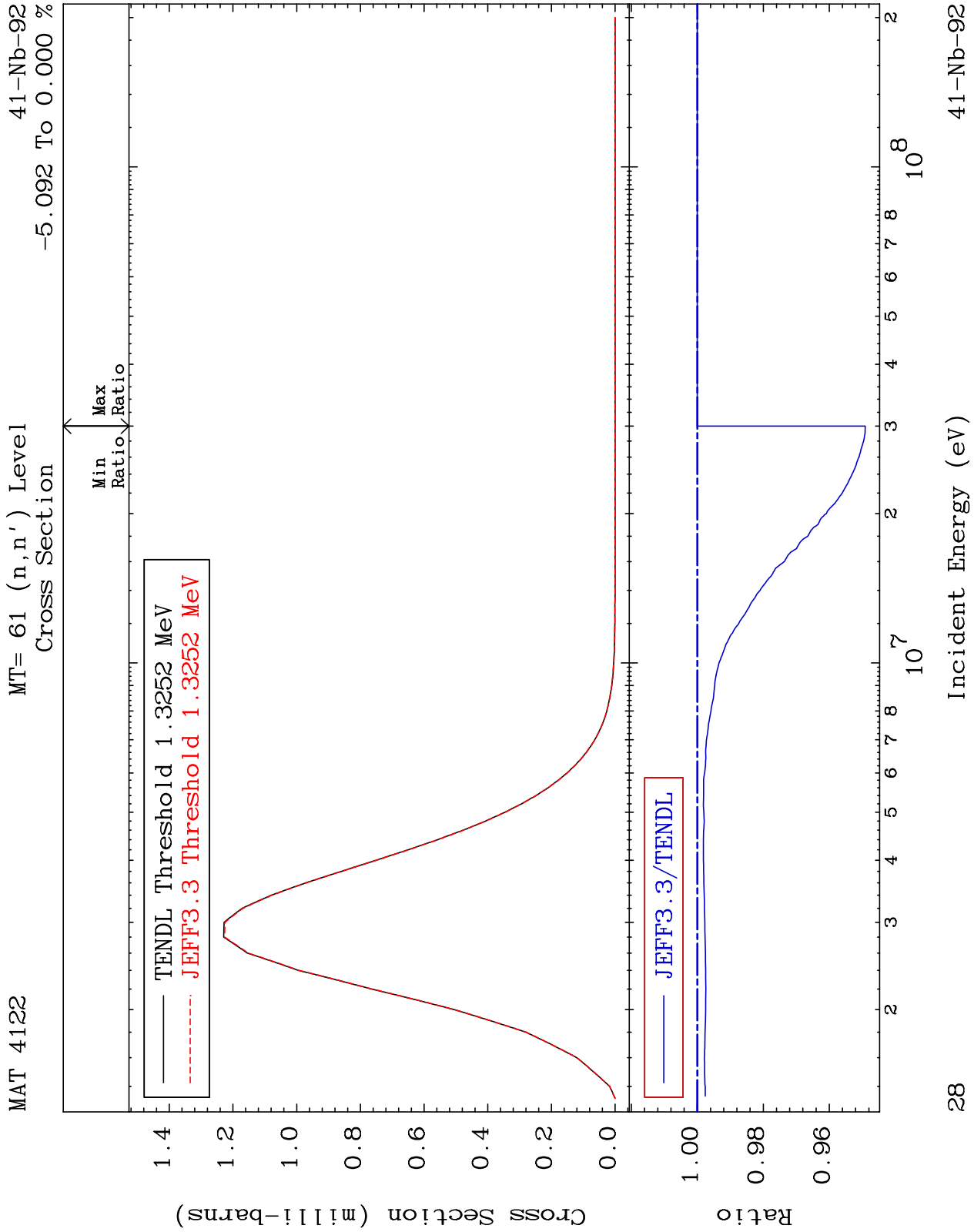
Cross Section

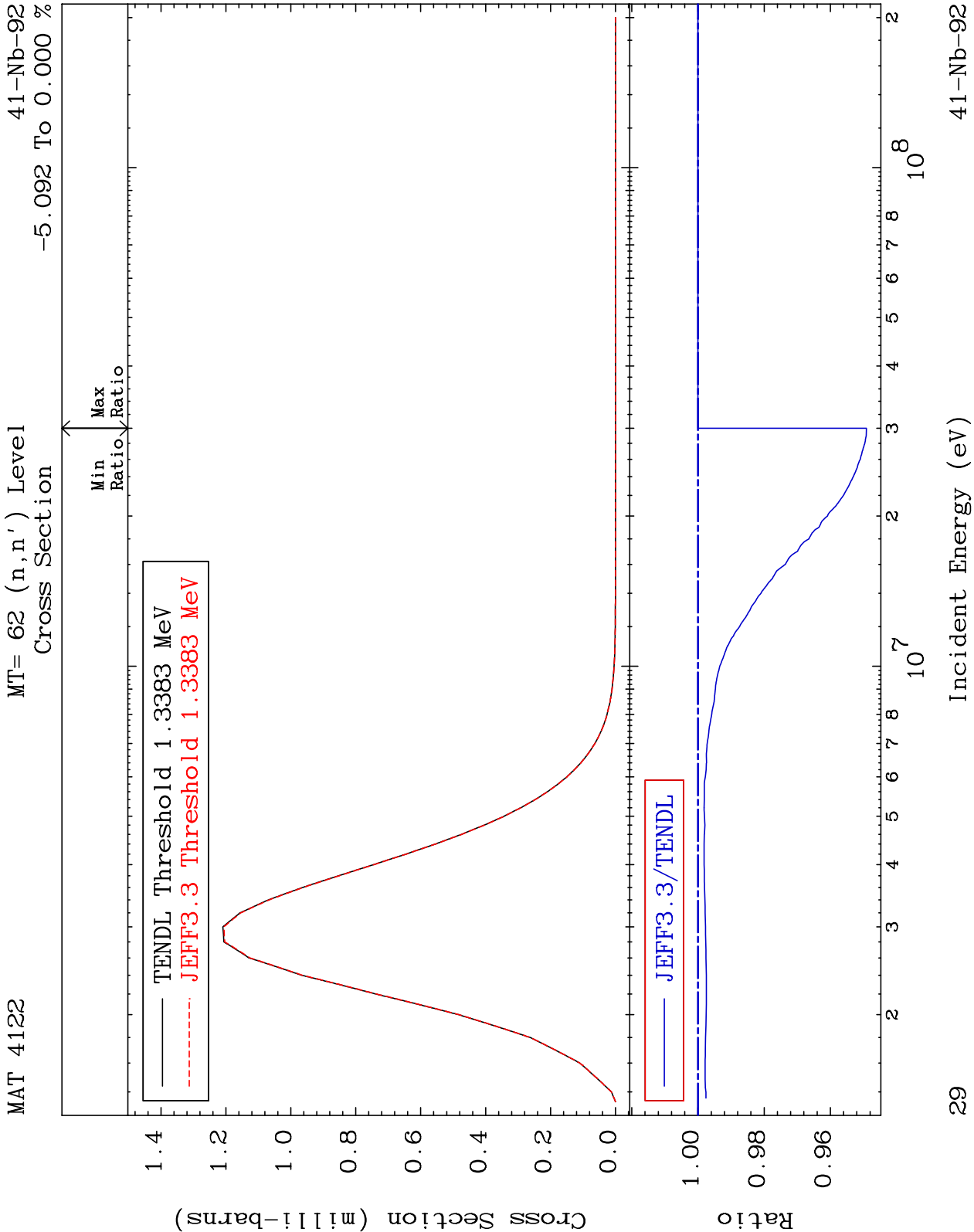
-5.092 To 0.000 %







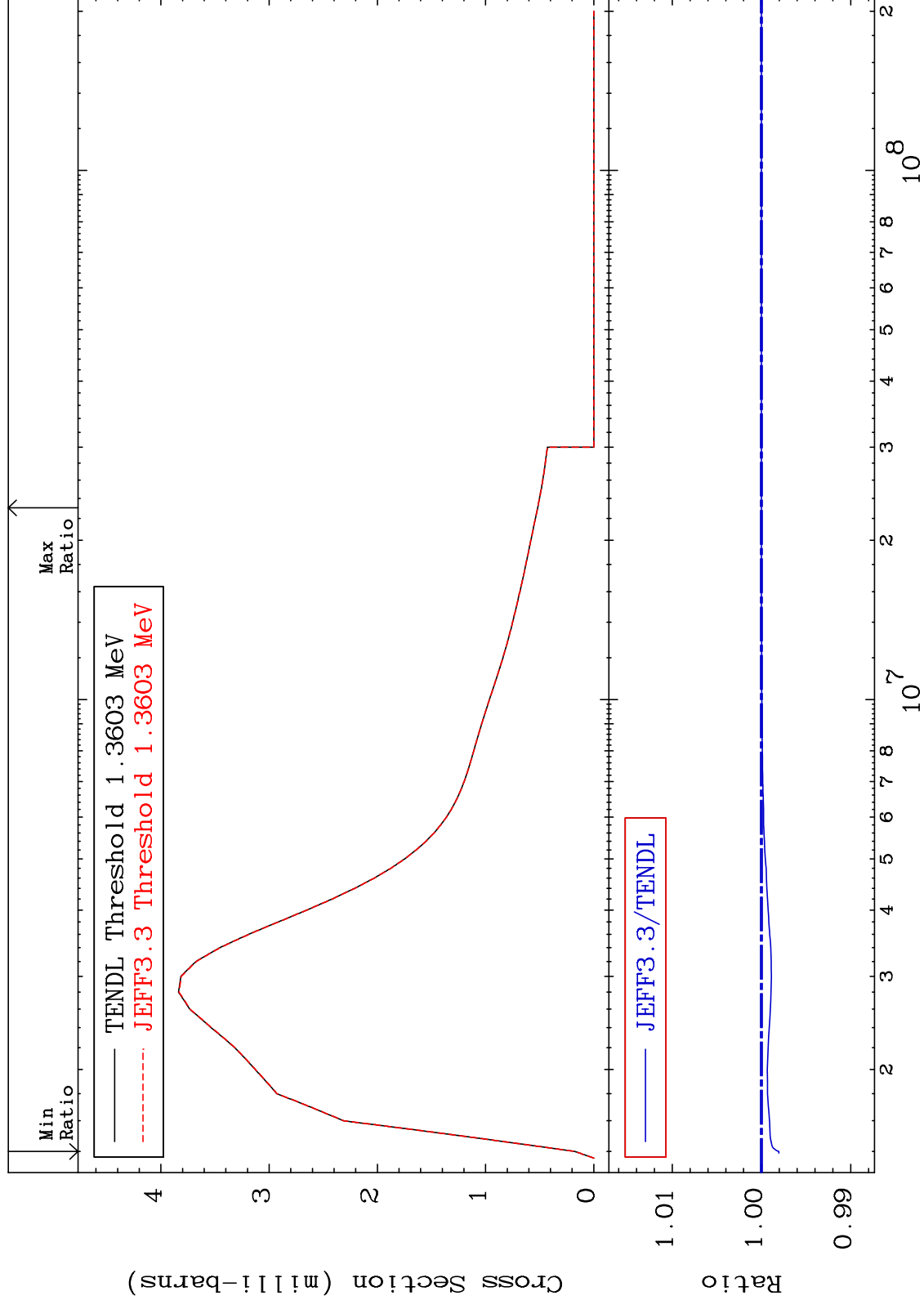




MAT 4122

MT= 63 (n, n') Level
Cross Section

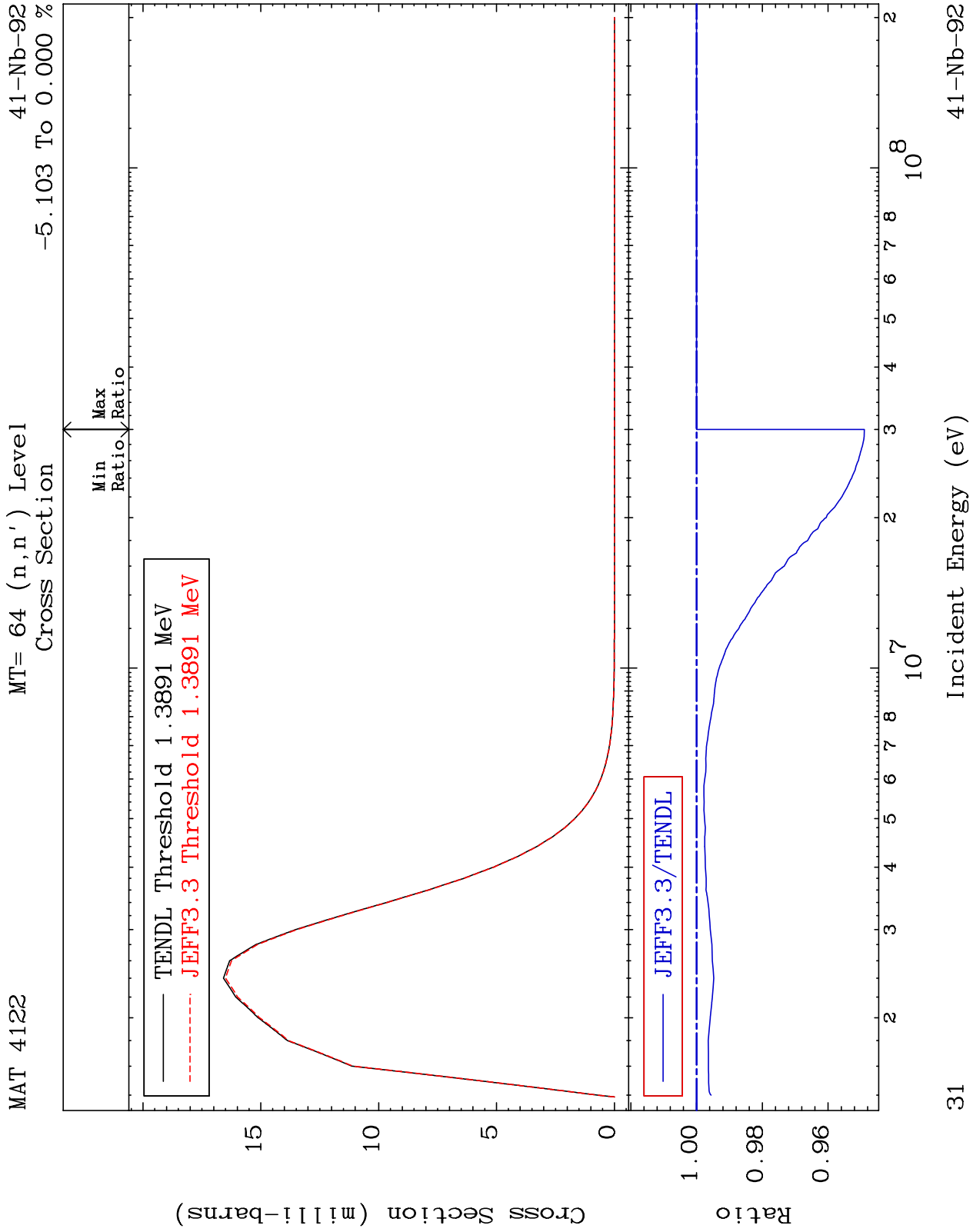
41-Nb-92
-0.195 To 0.000 %

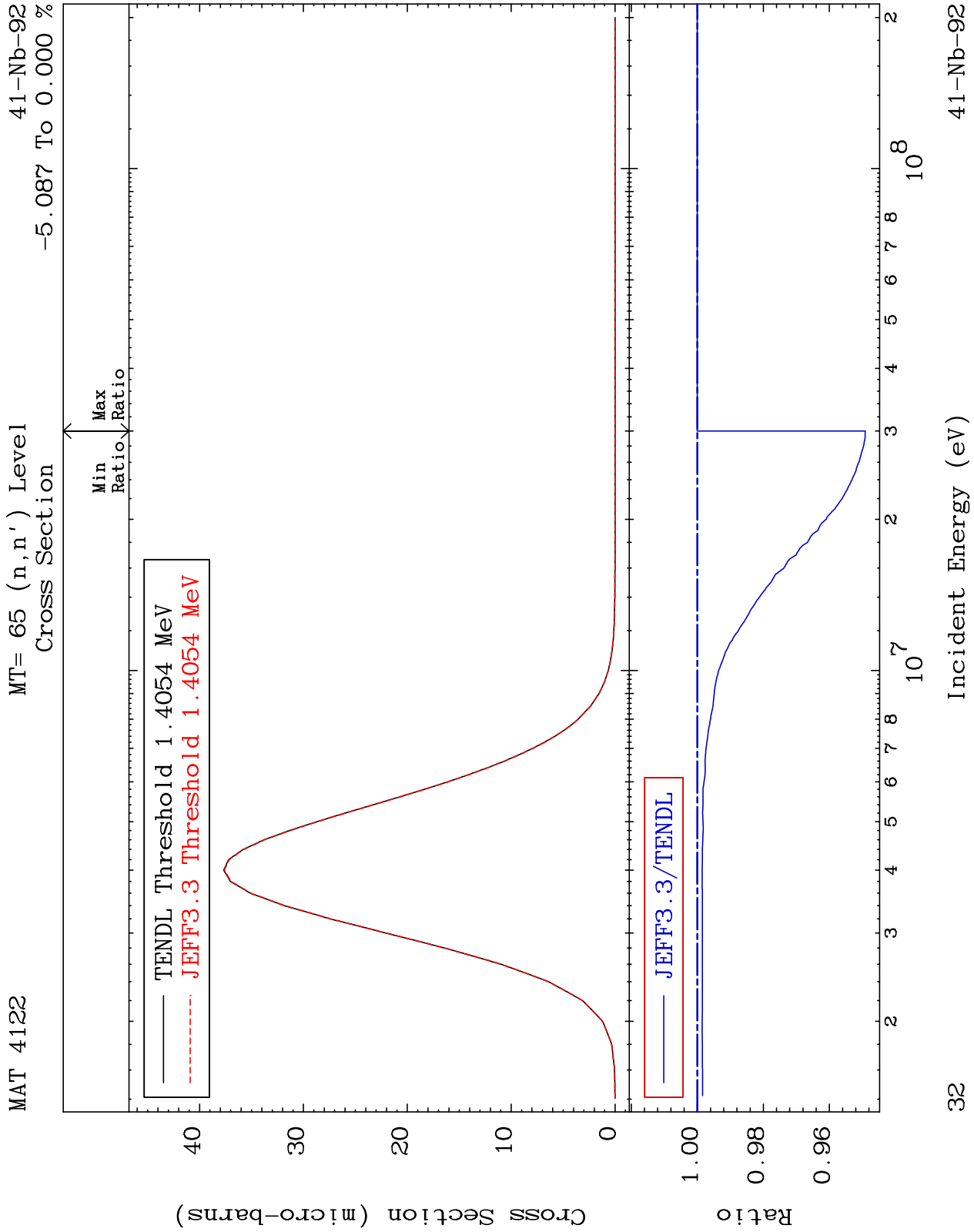


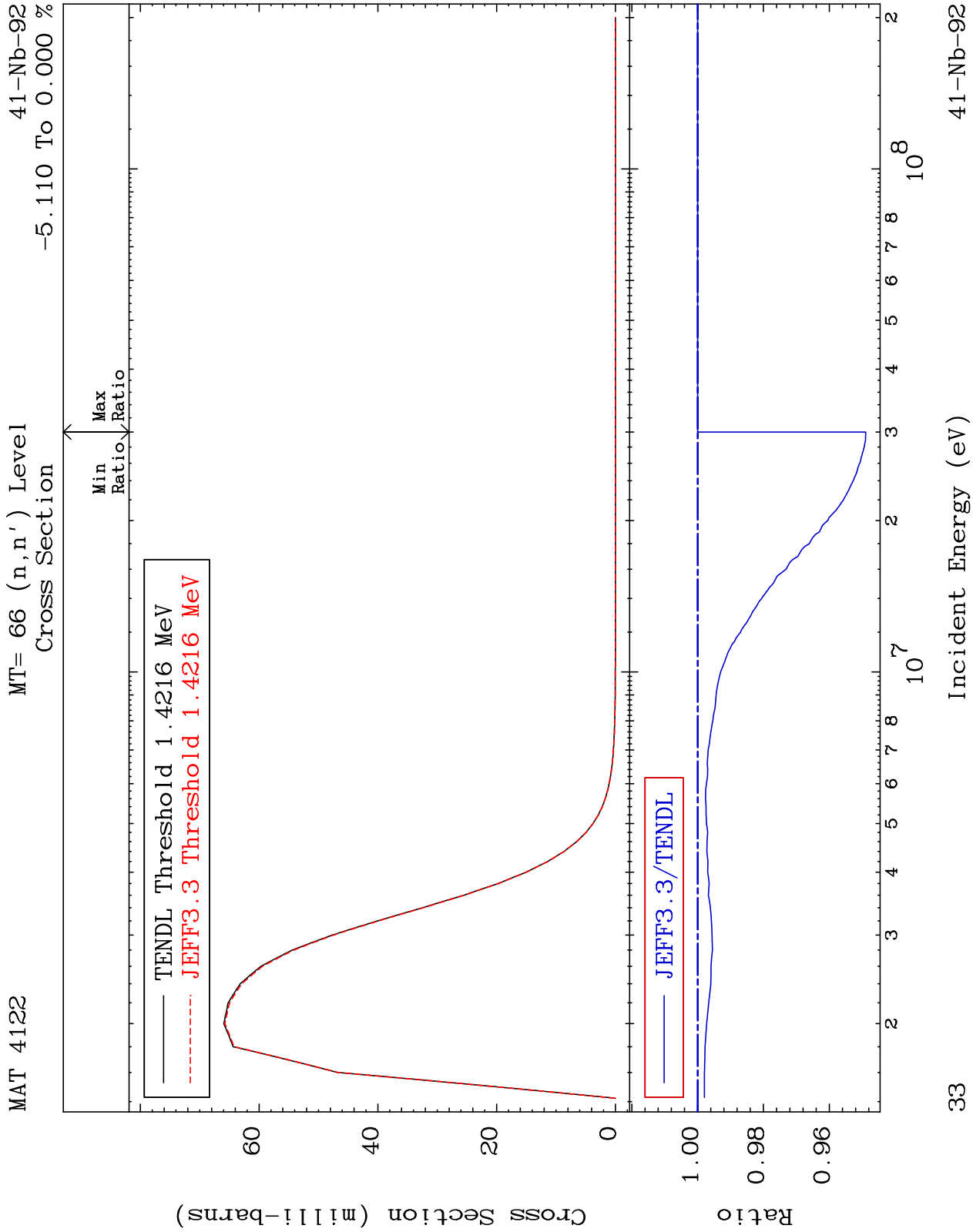
30

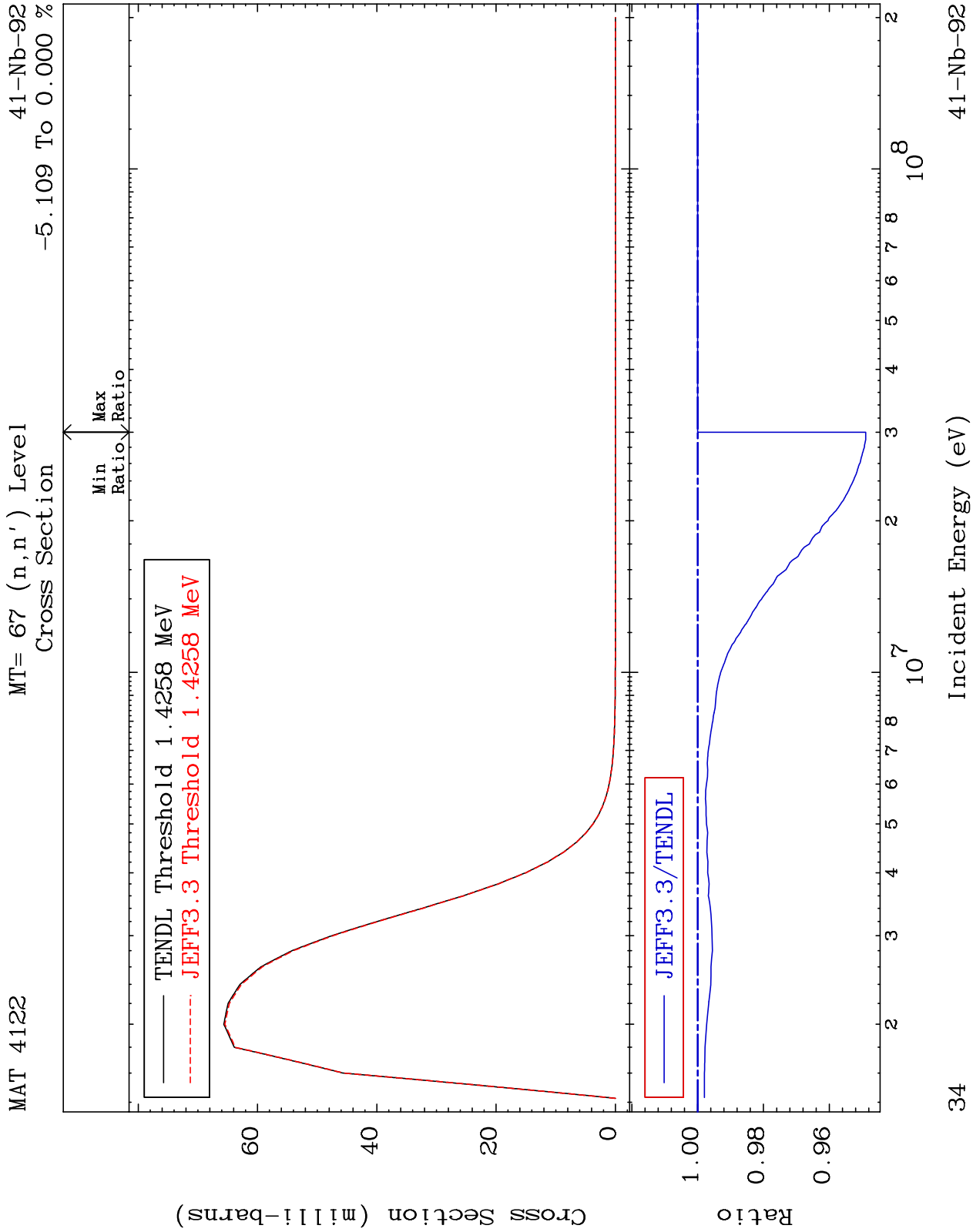
Incident Energy (eV)

41-Nb-92





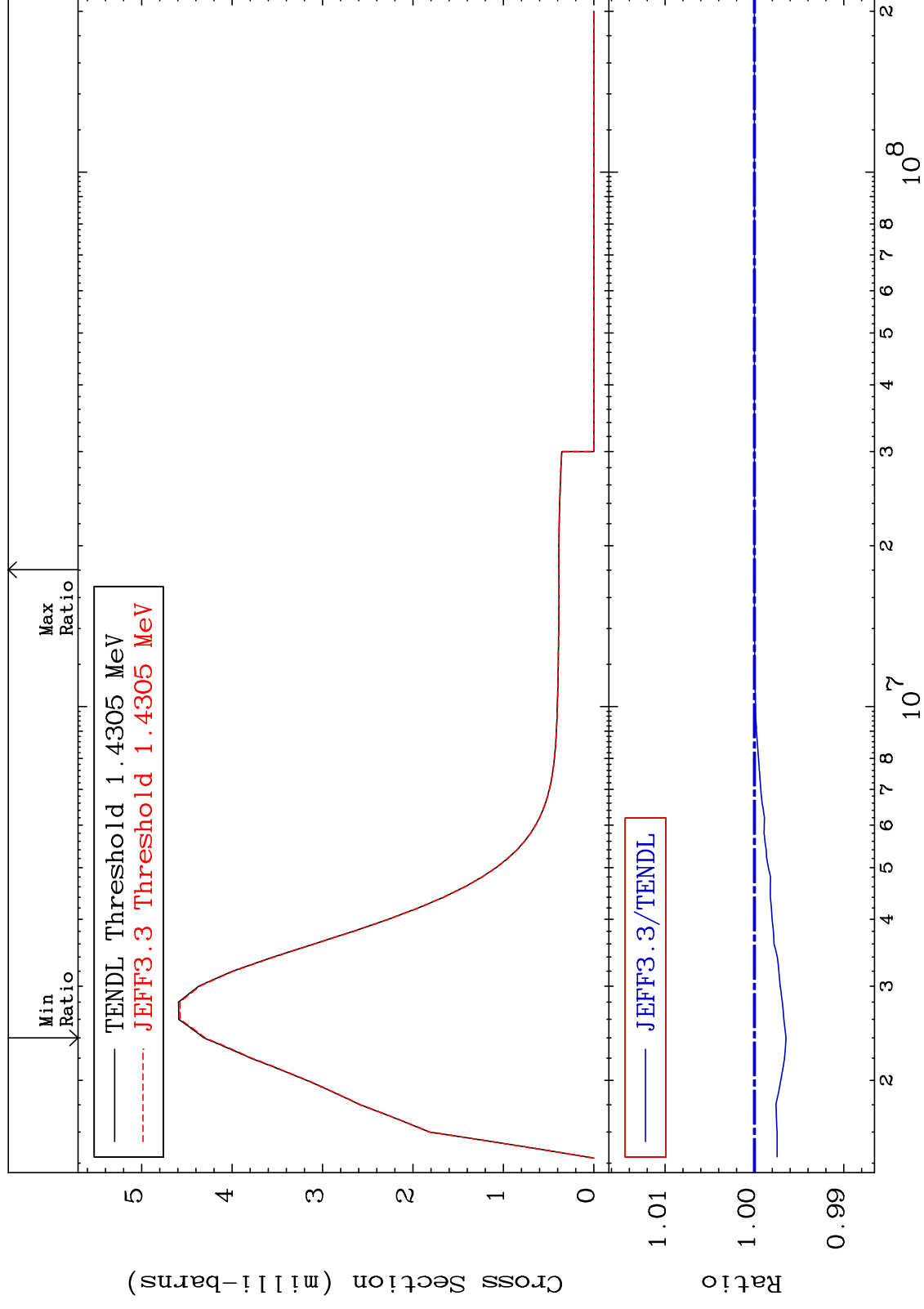




MAT 4122

MT= 68 (n, n') Level
Cross Section

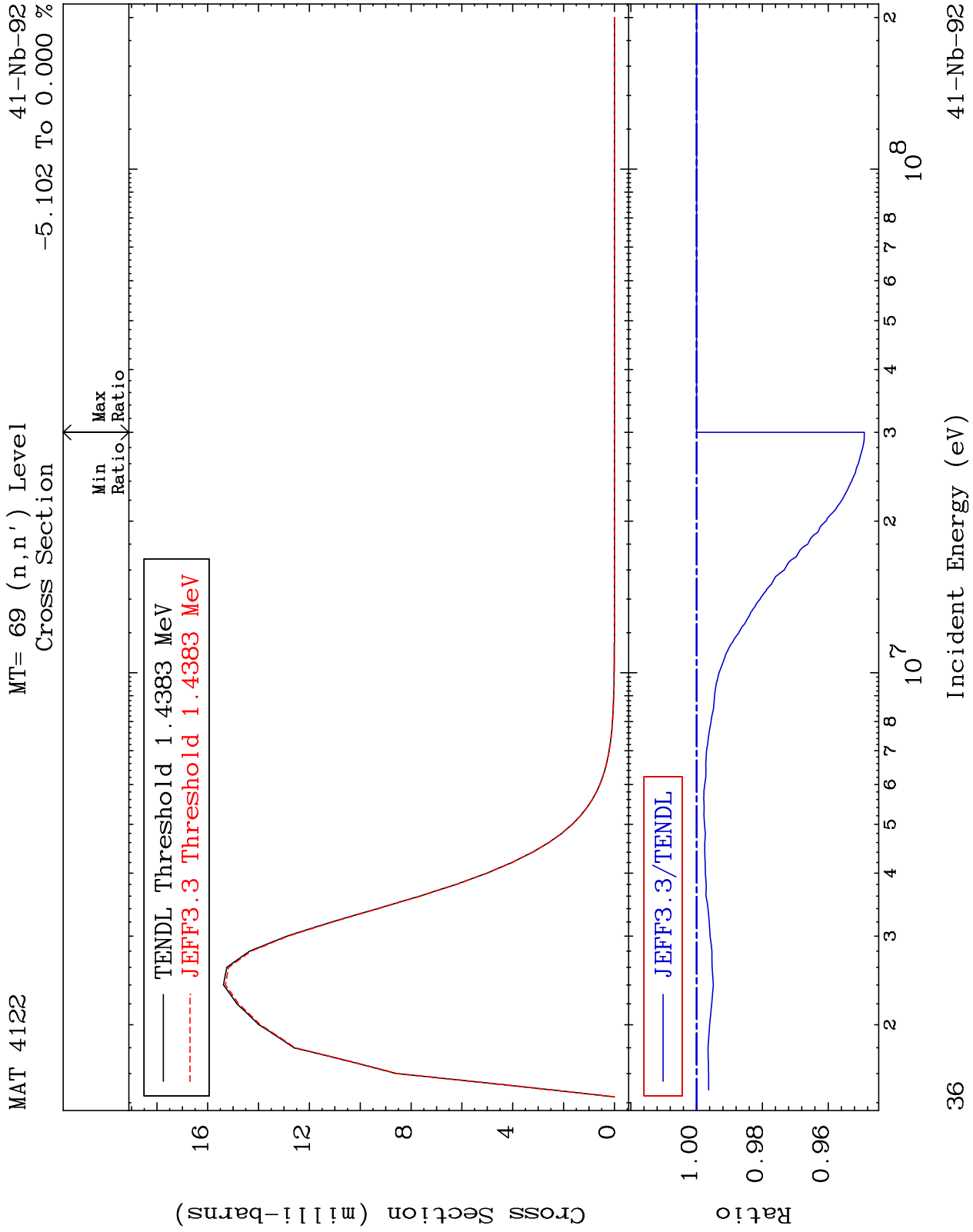
41-Nb-92
-0.353 To 0.000 %

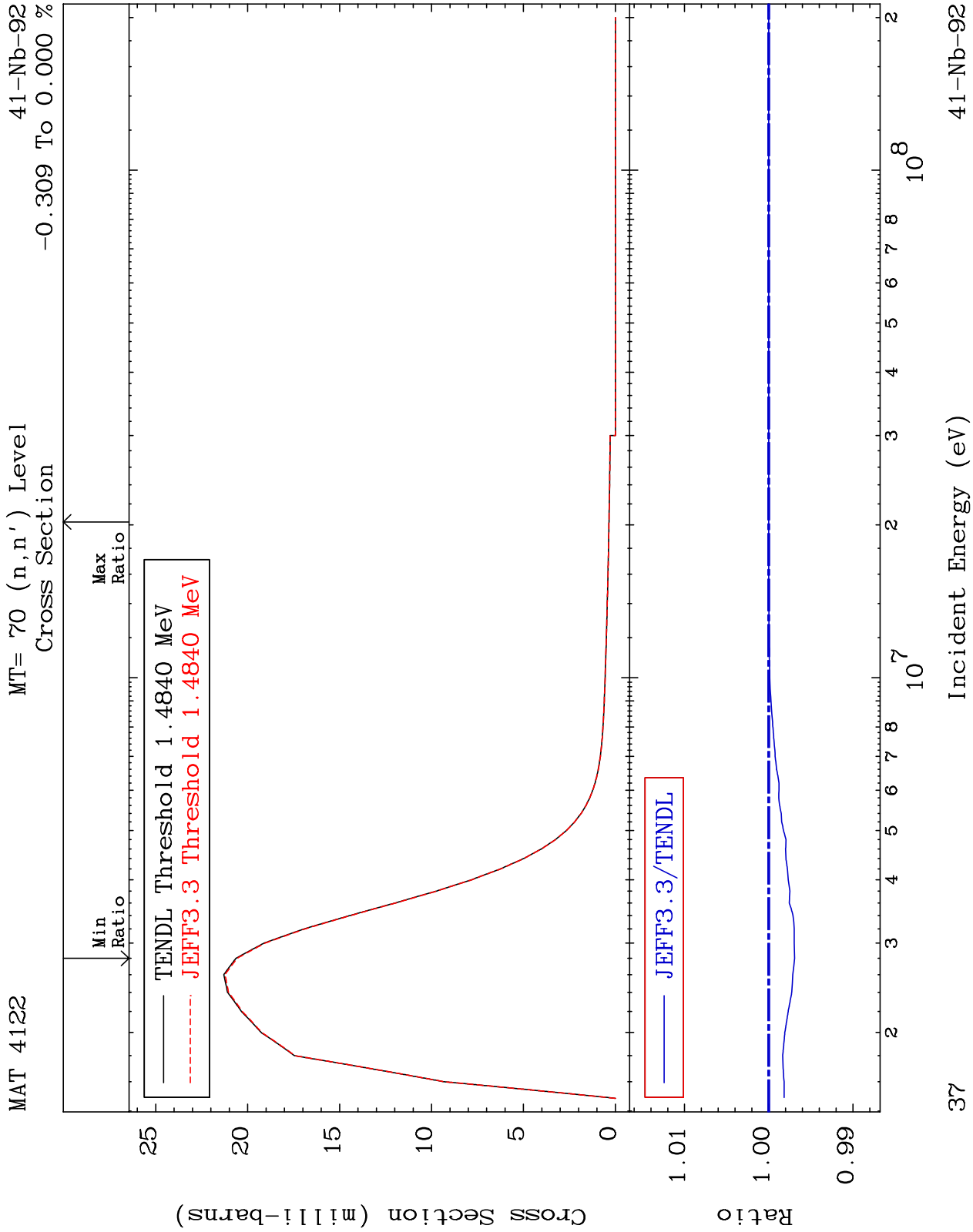


35

Incident Energy (eV)

41-Nb-92

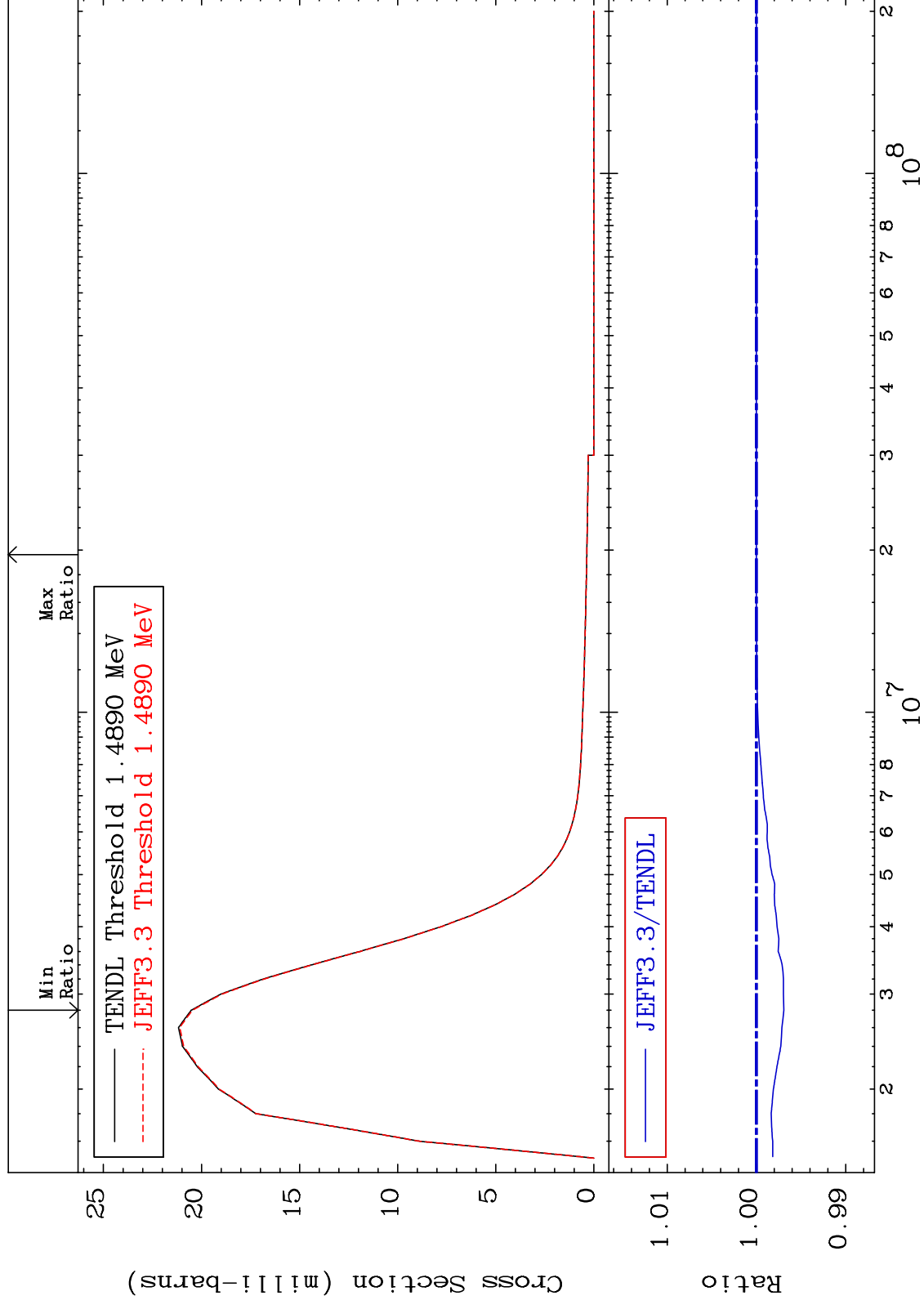




MAT 4122

MT= 71 (n, n') Level
Cross Section

41-Nb-92
-0.308 To 0.000 %



38

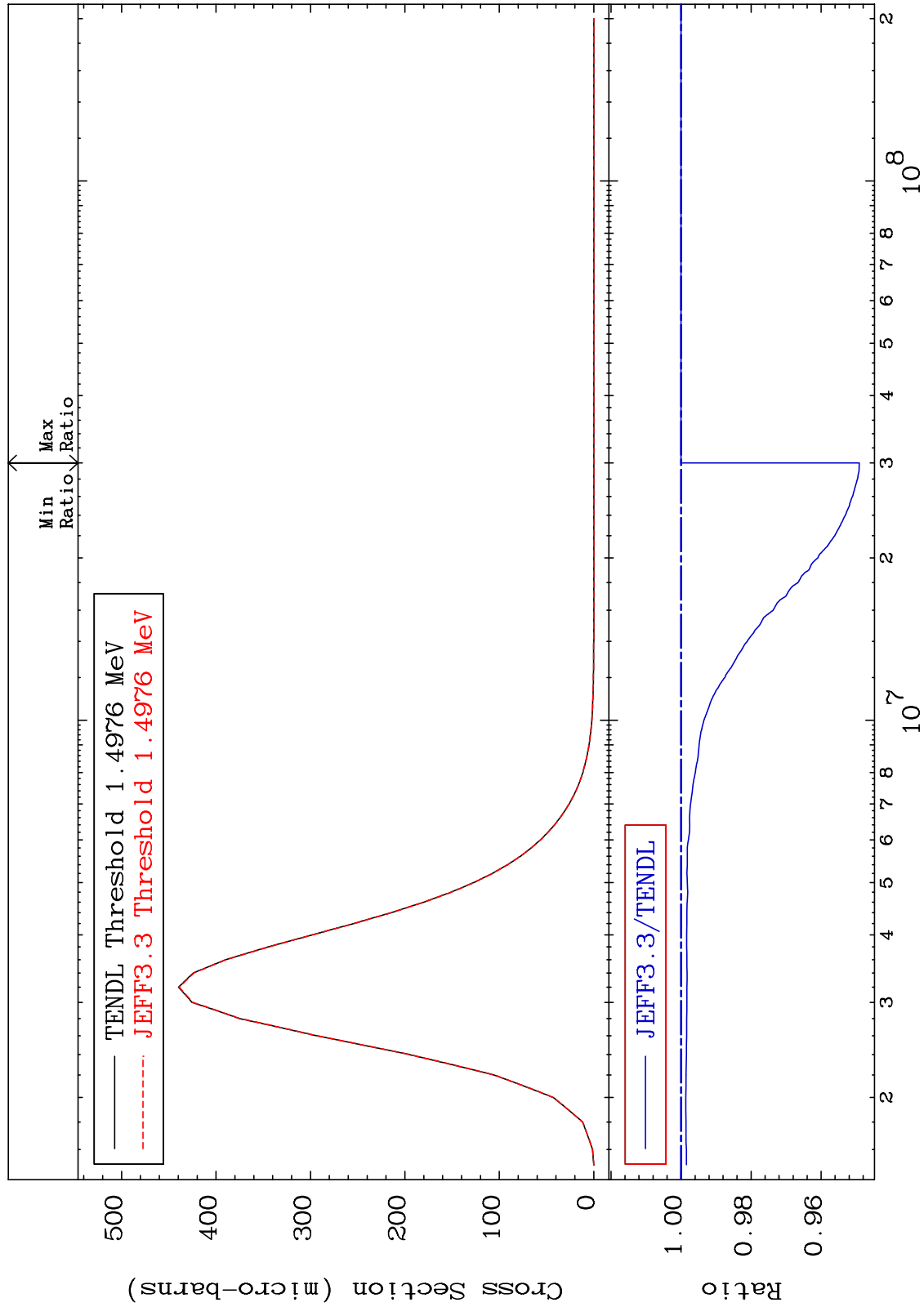
Incident Energy (eV)

41-Nb-92

MAT 4122

MT= 72 (n,n') Level
Cross Section

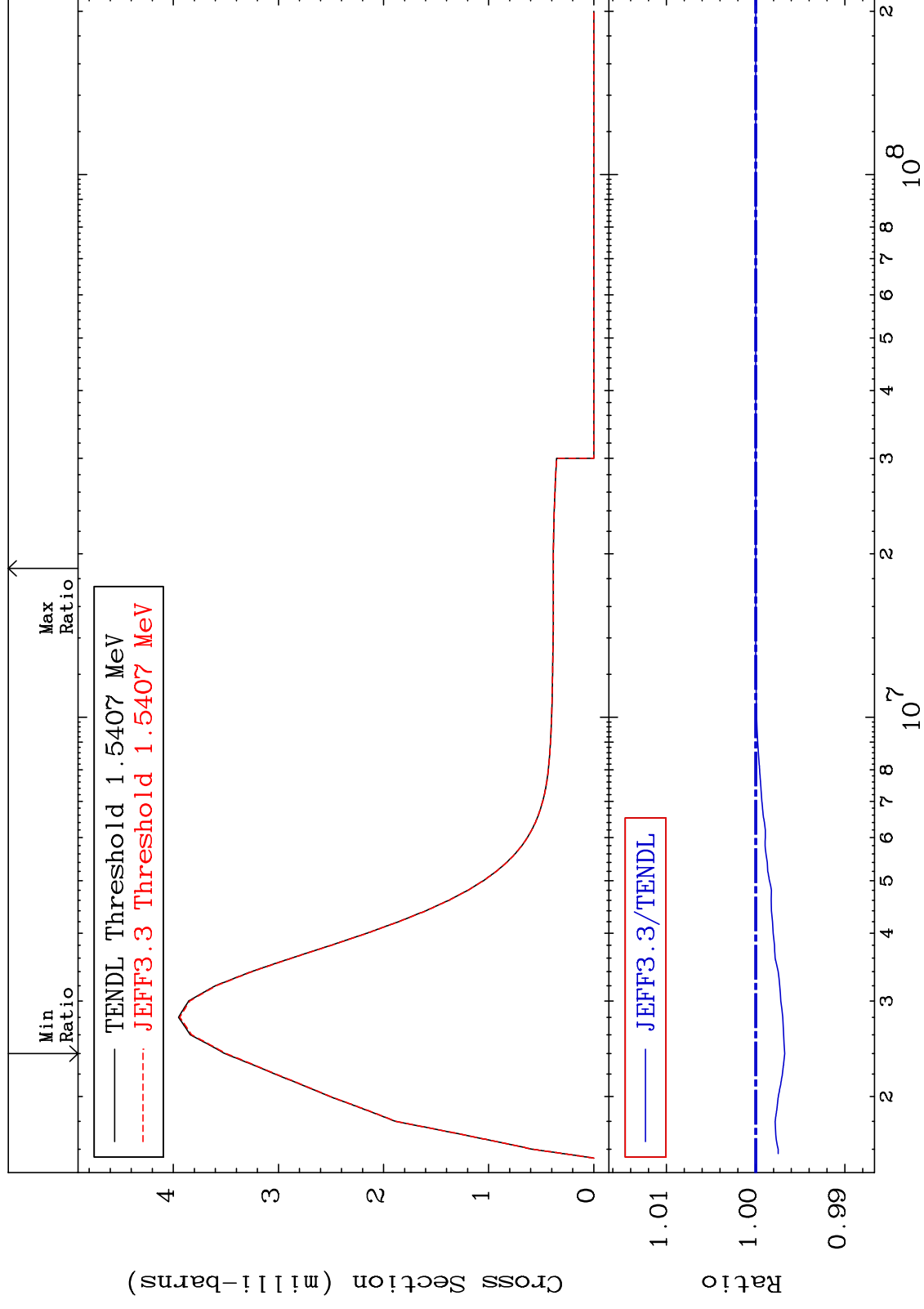
41-Nb-92
-5.089 To 0.000 %



MAT 4122

MT= 73 (n, n') Level
Cross Section

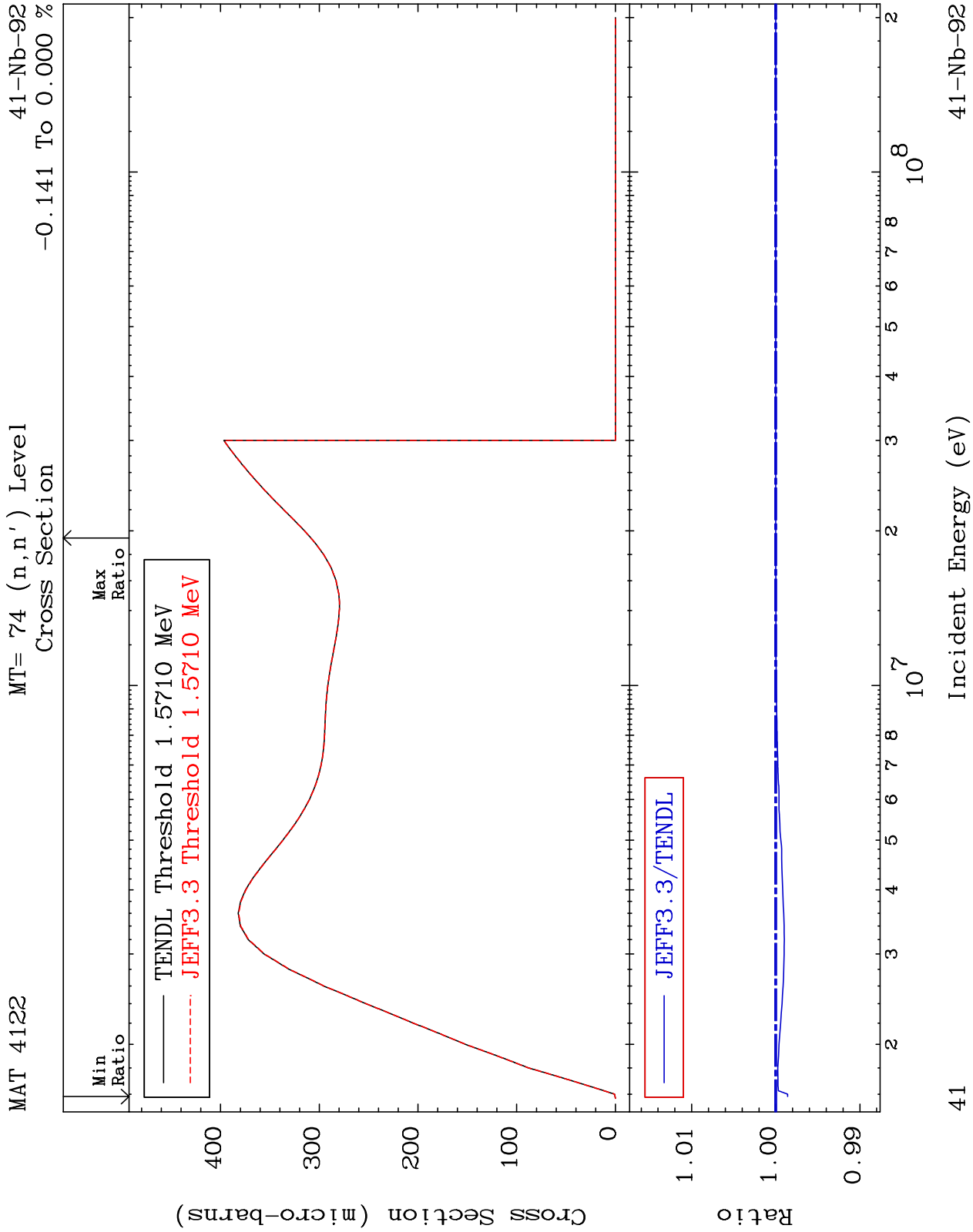
41-Nb-92
-0.325 To 0.000 %

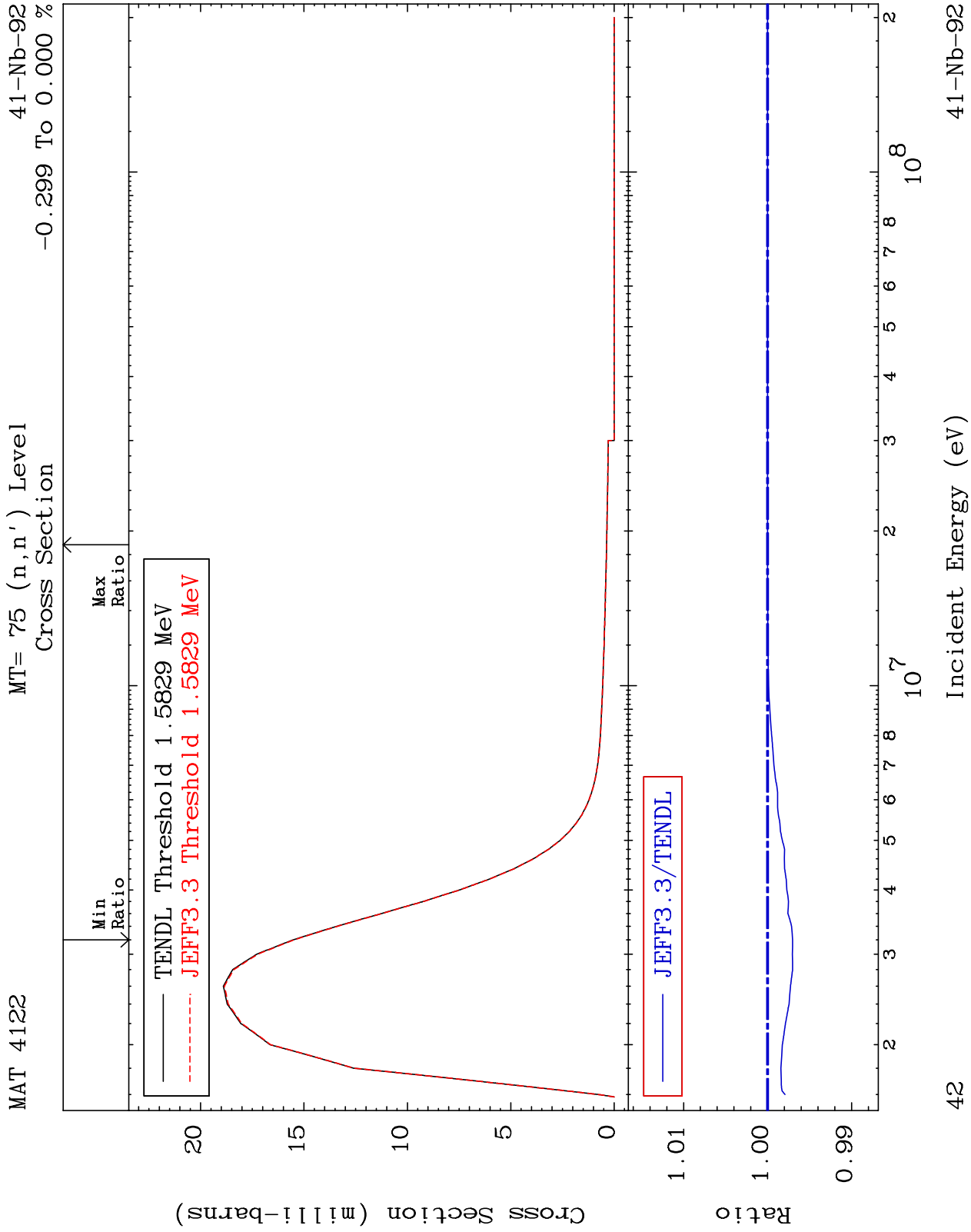


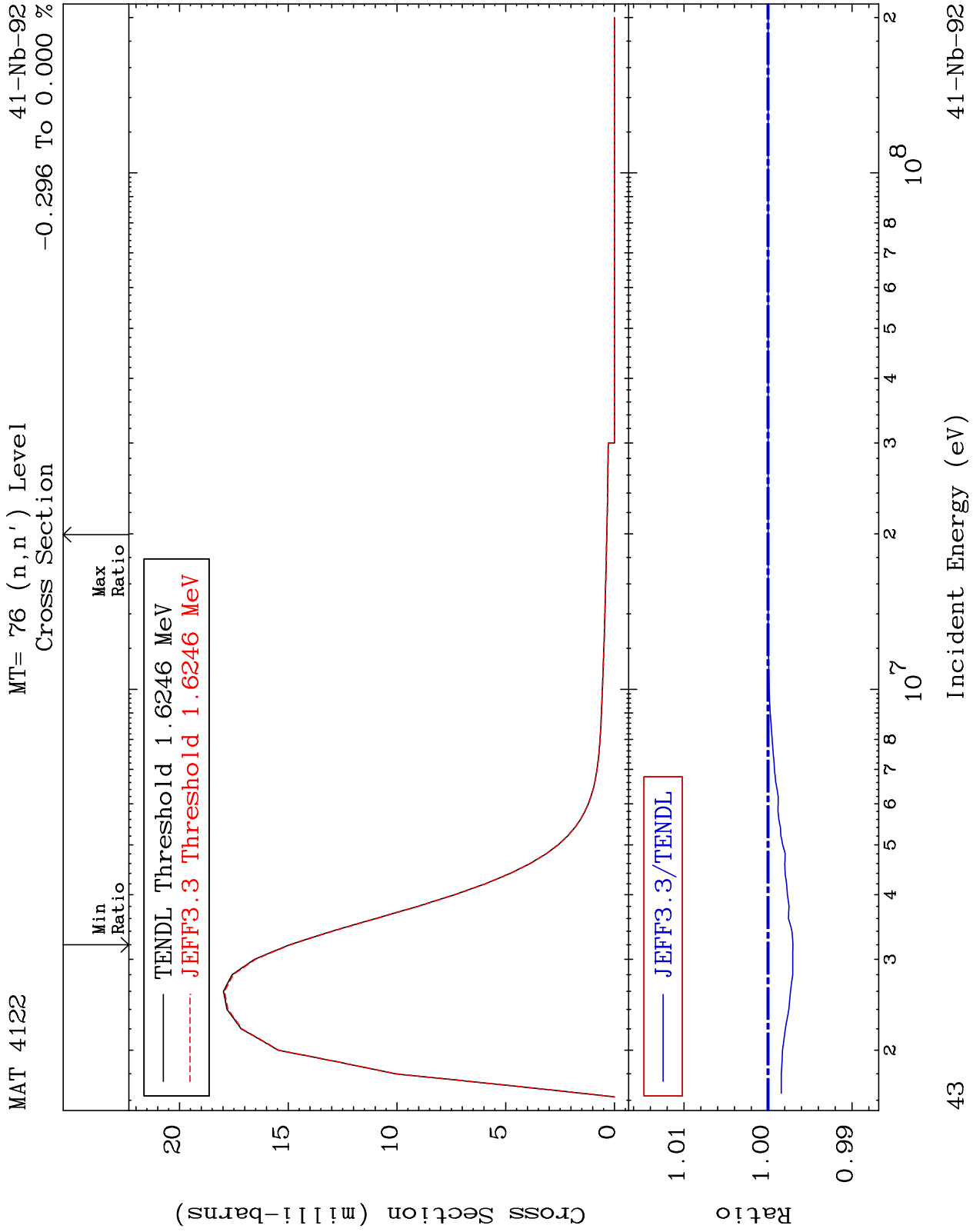
40

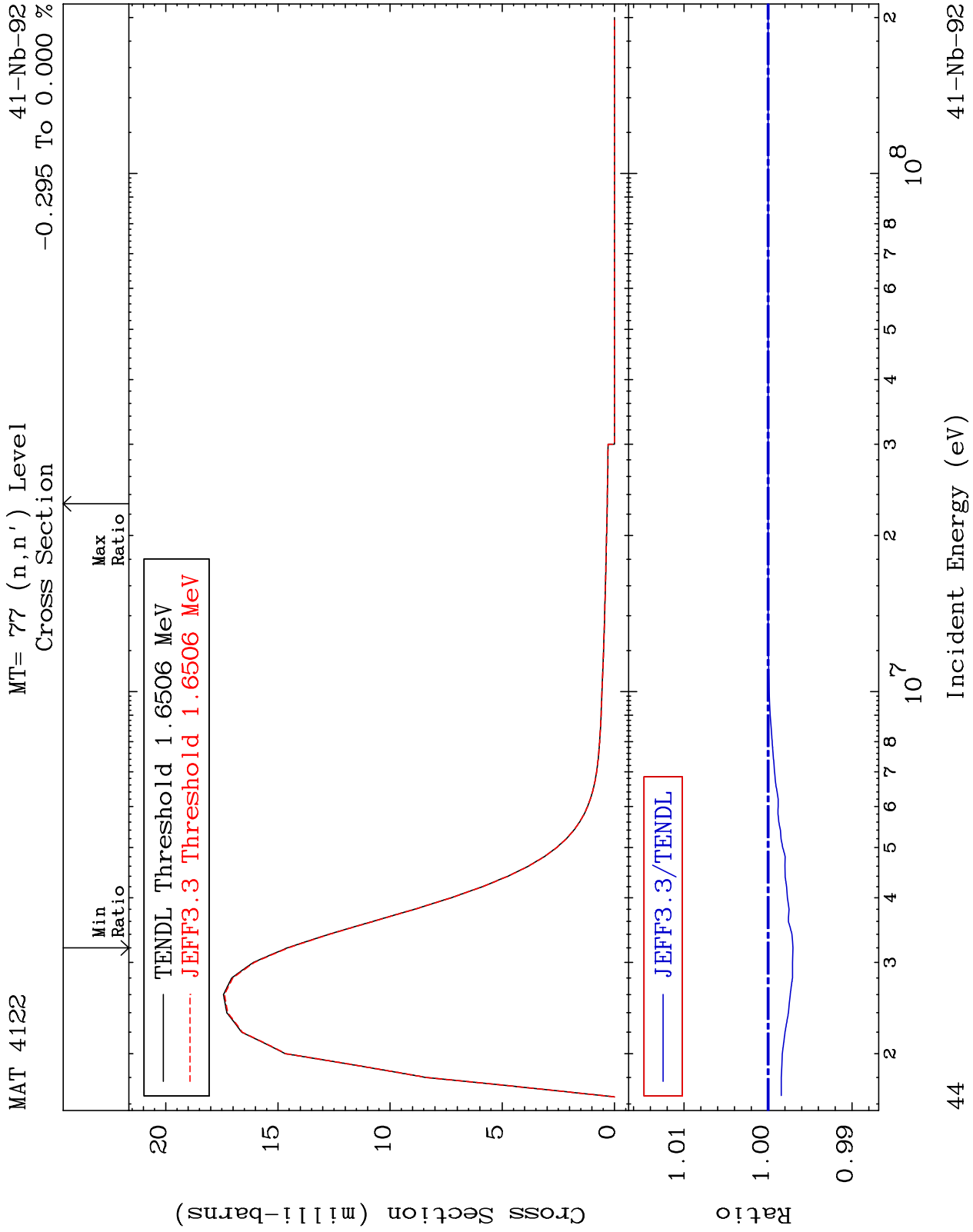
Incident Energy (eV)

41-Nb-92





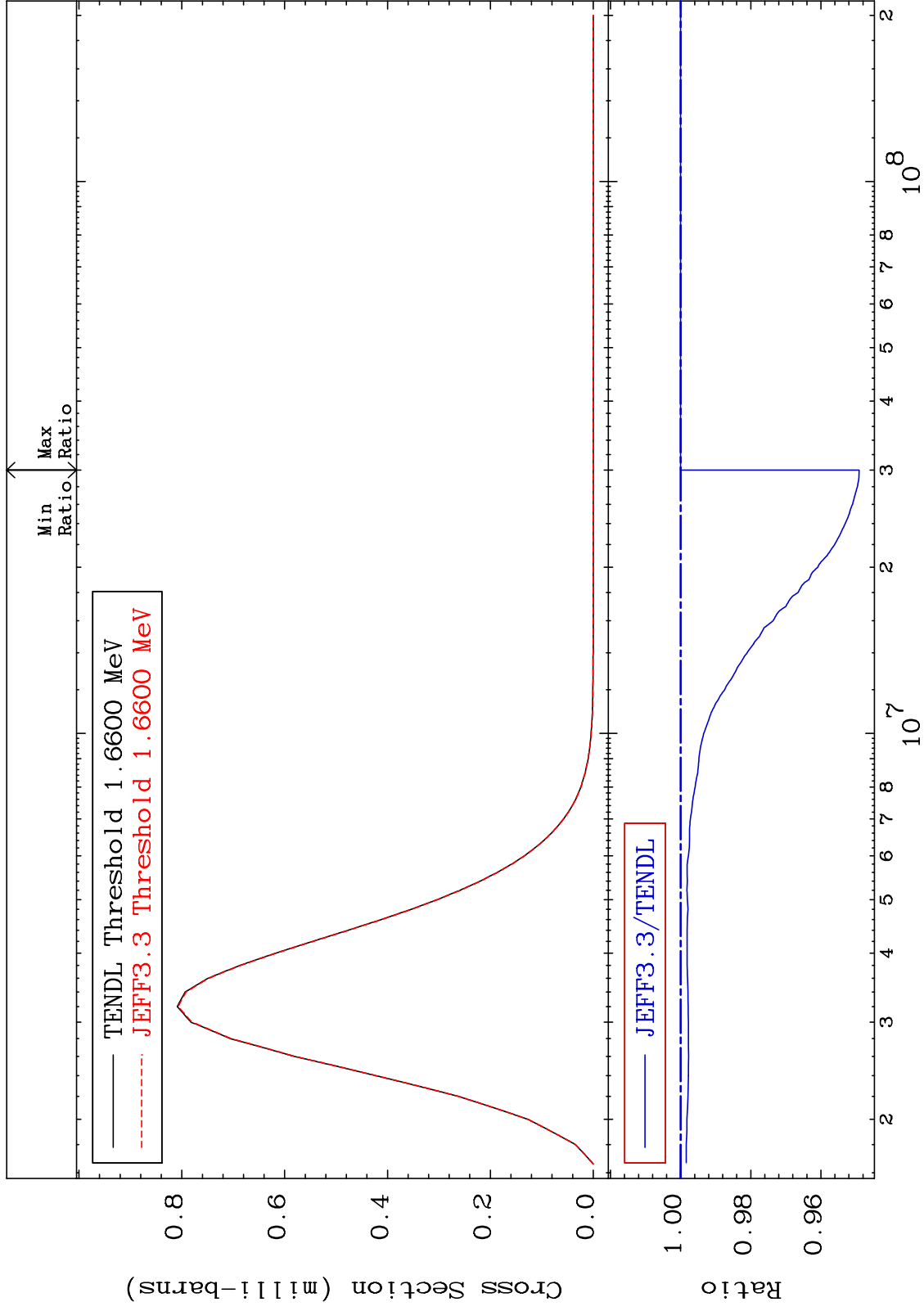




MAT 4122

MT= 78 (n,n') Level
Cross Section

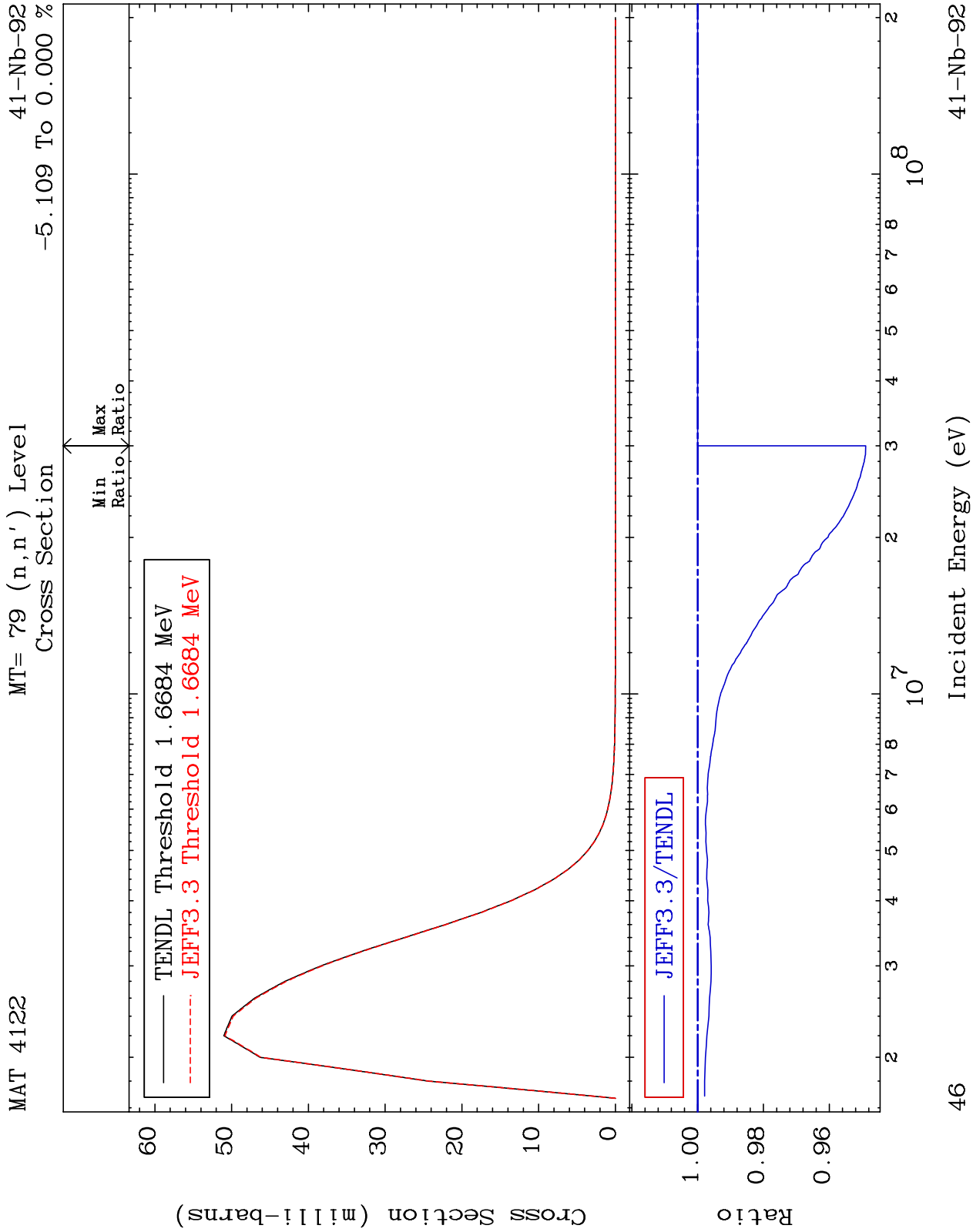
41-Nb-92
-5.091 To 0.000 %

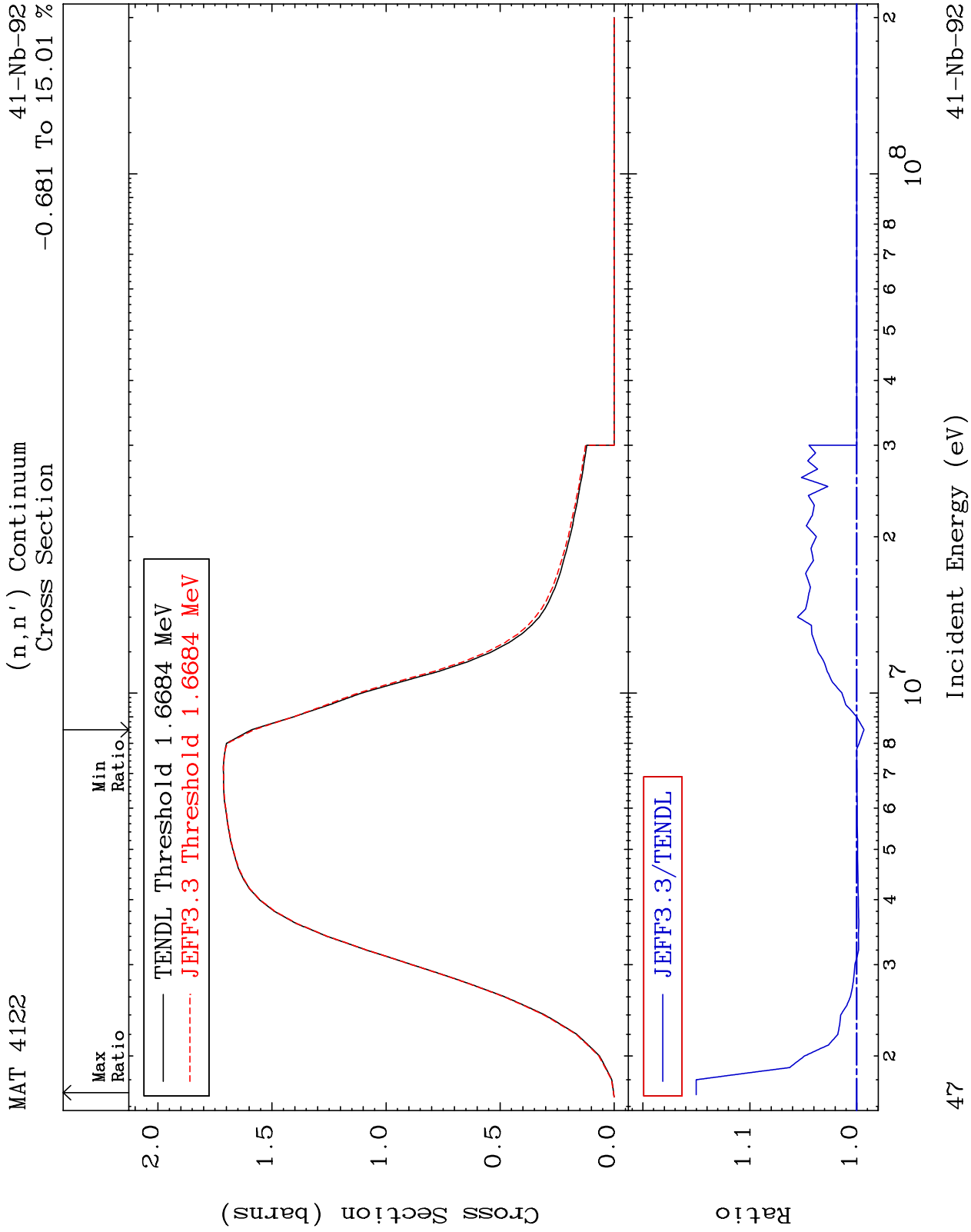


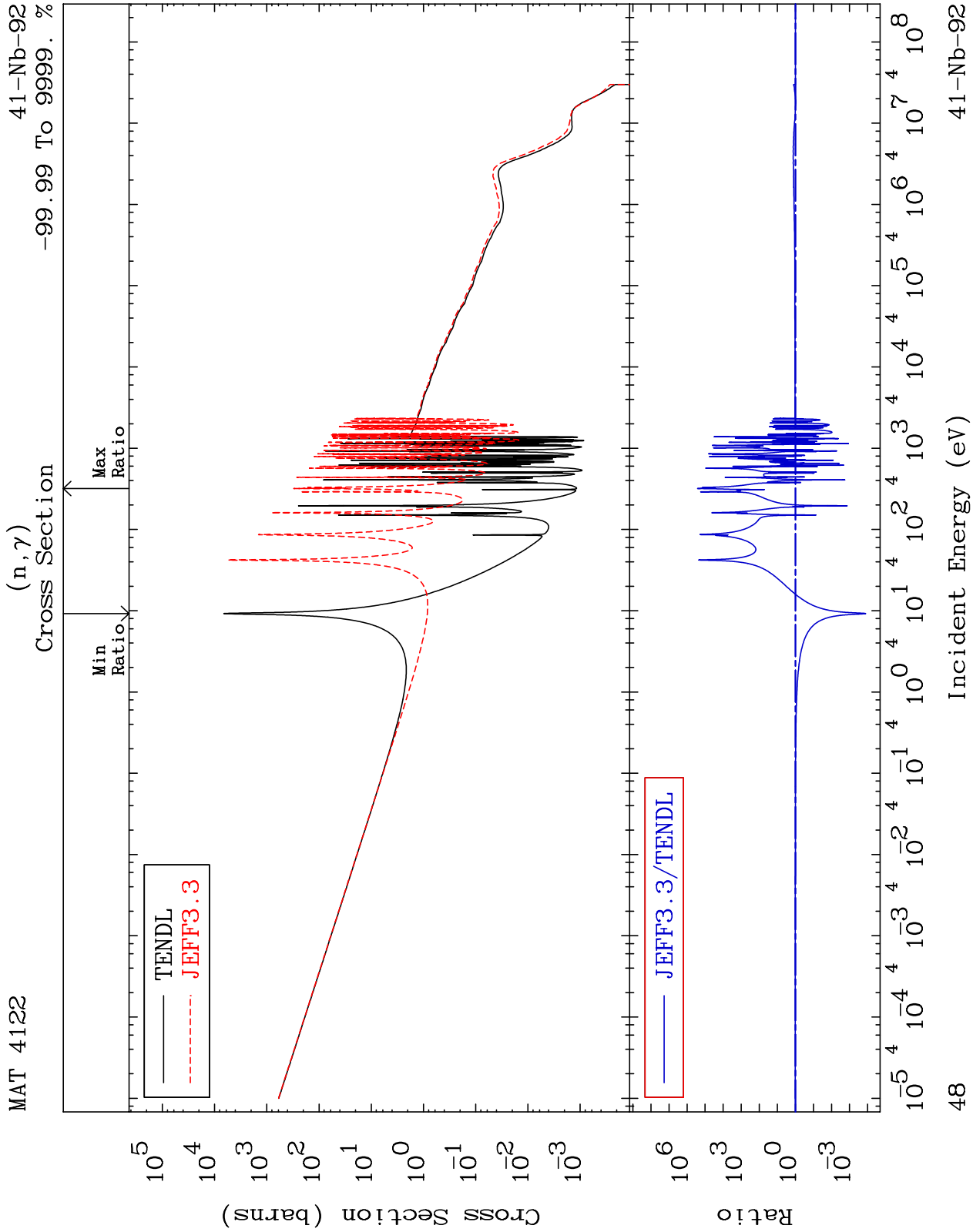
45

Incident Energy (eV)

41-Nb-92







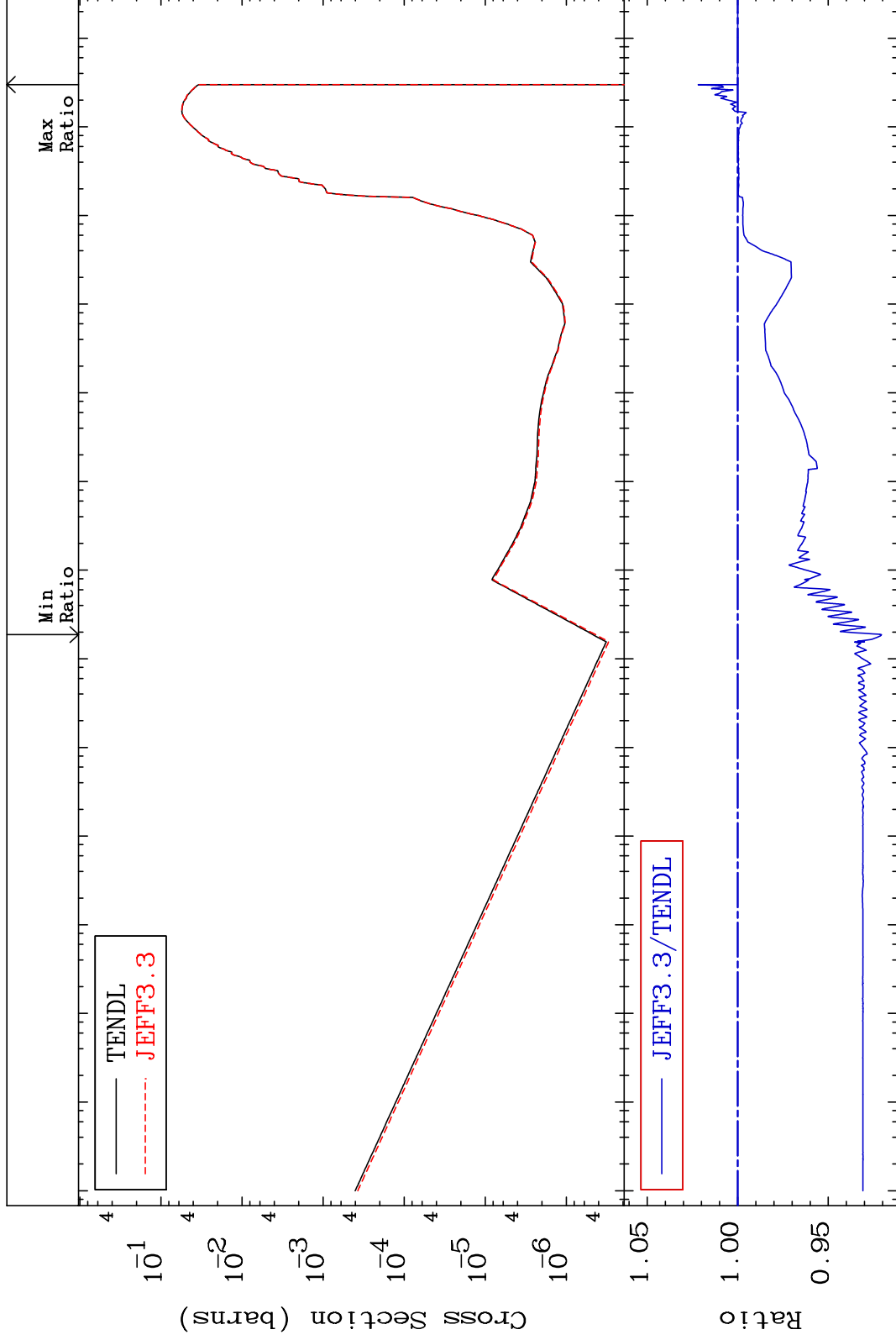
MAT 4122

(n,p)

41-Nb-92

Cross Section

-7.941 To 2.175 %



Incident Energy (eV)

41-Nb-92

49

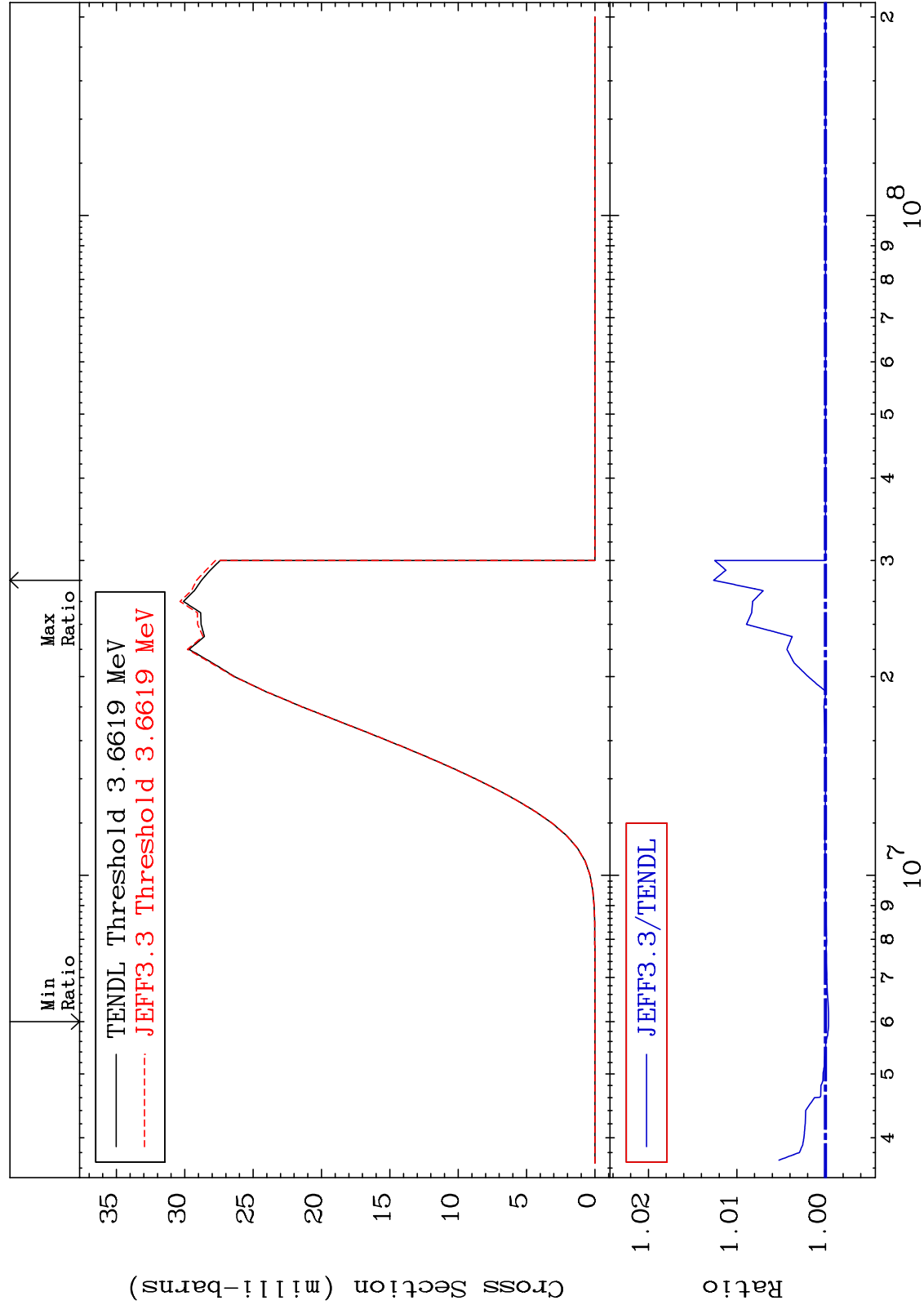
MAT 4122

(n,d)

41-Nb-92

Cross Section

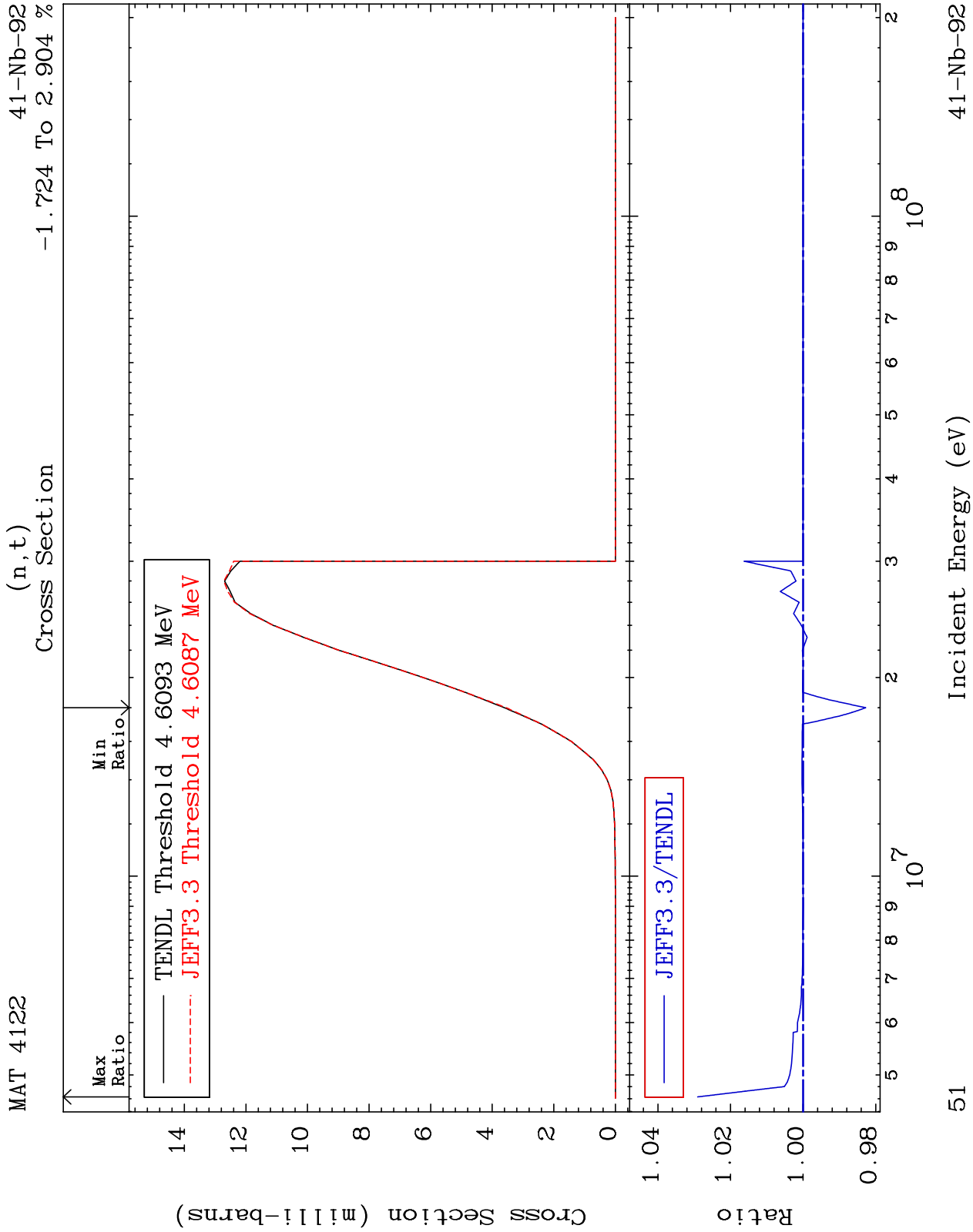
-0.036 To 1.263 %

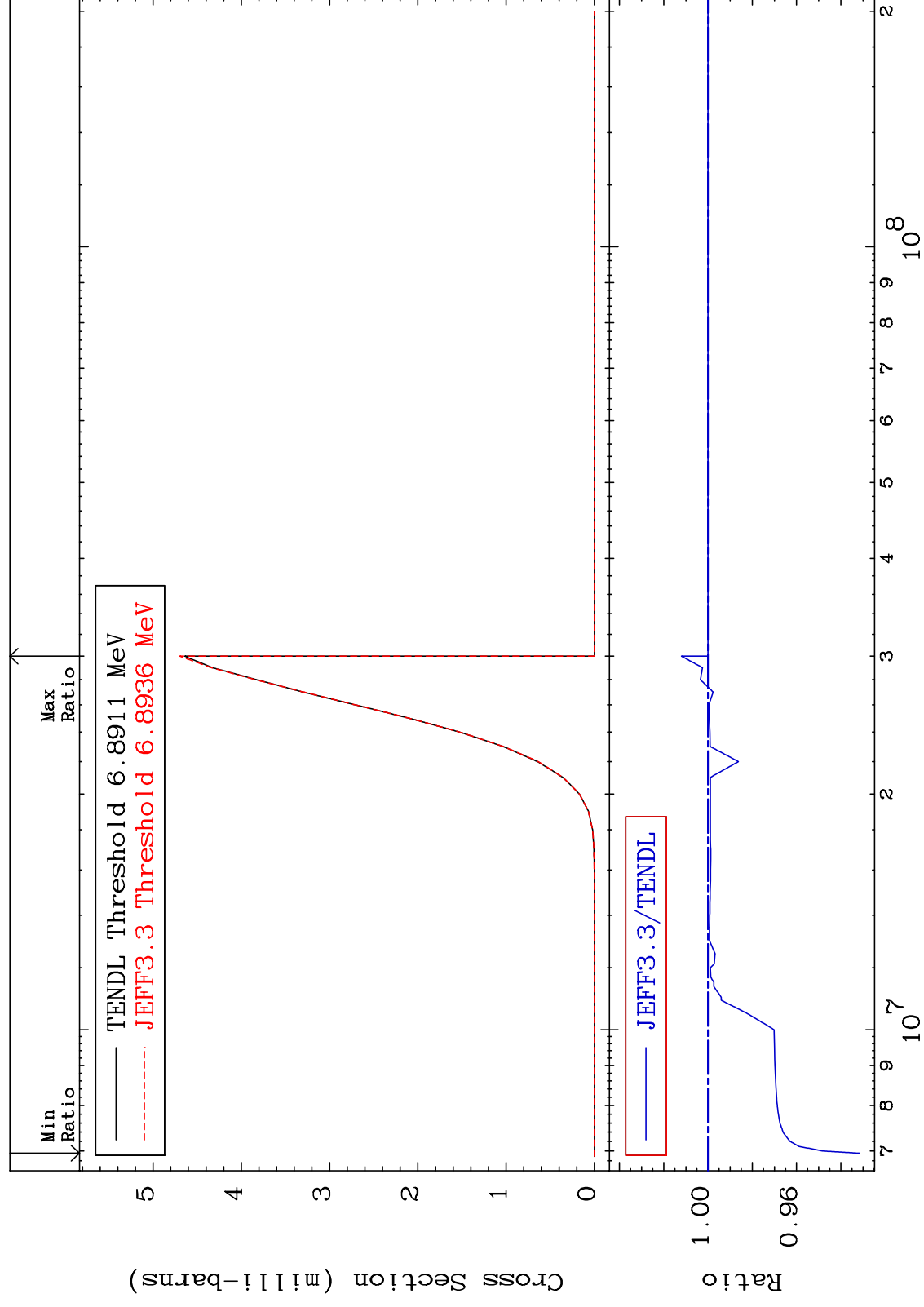


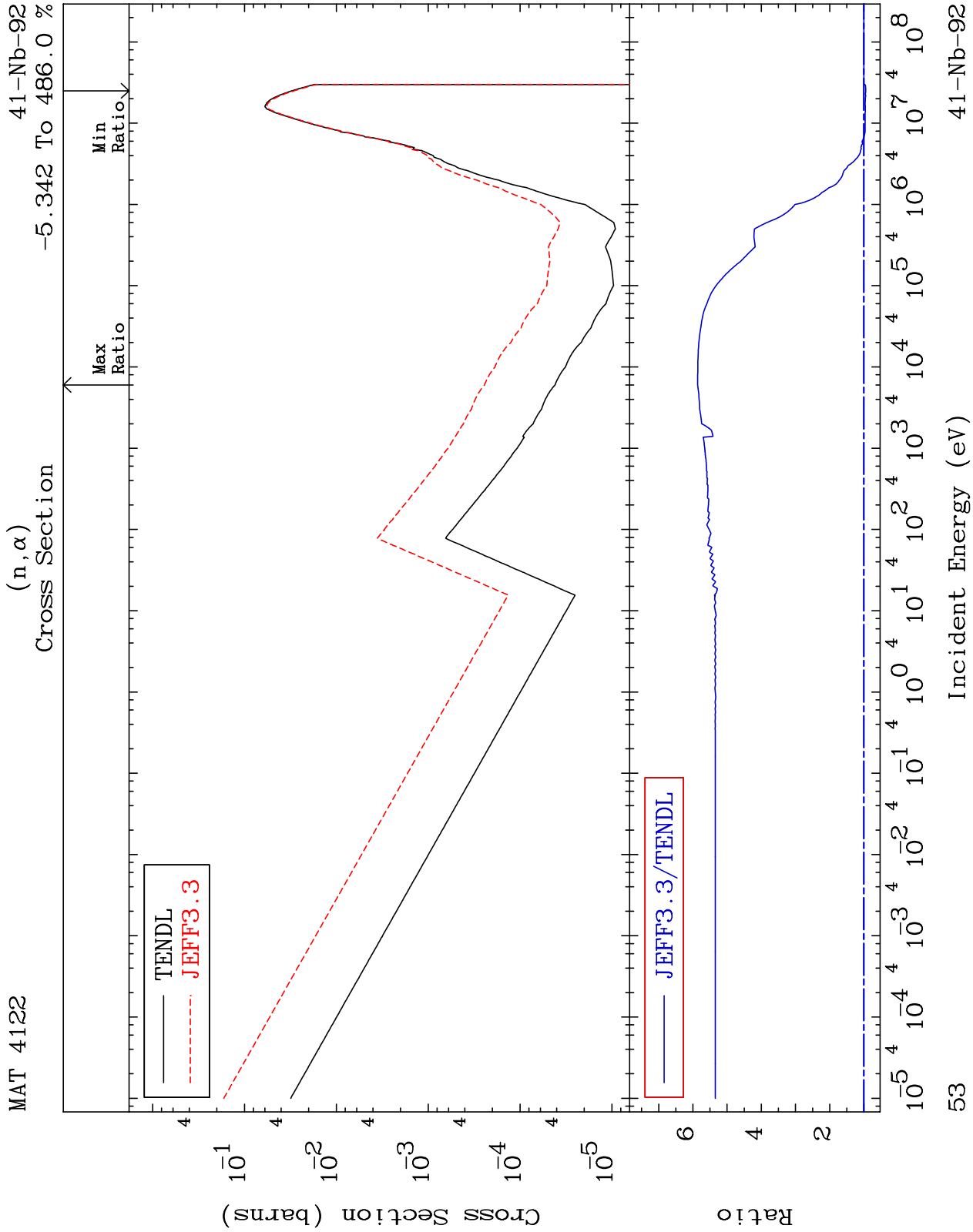
50

Incident Energy (eV)

41-Nb-92







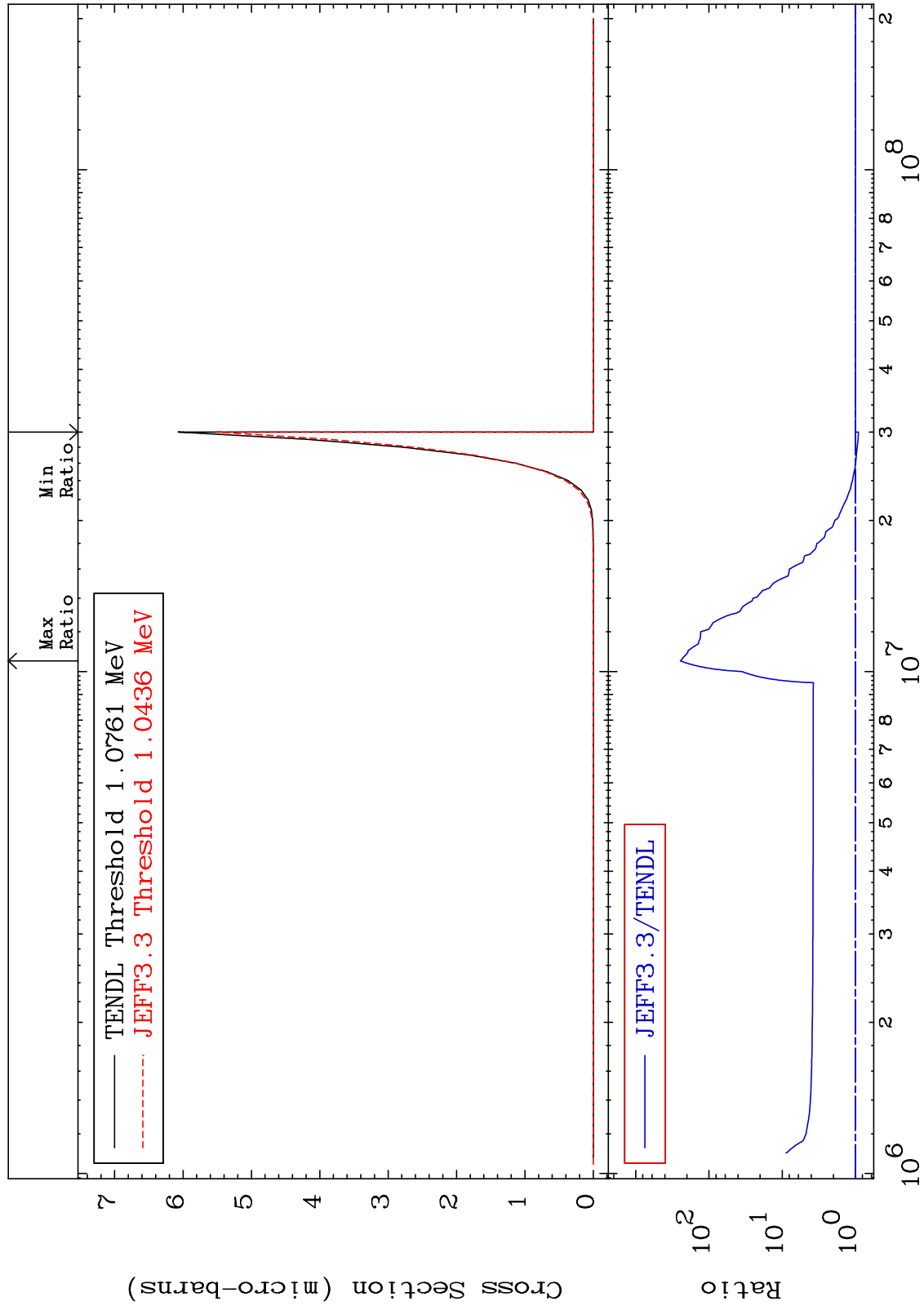
MAT 4122

(n,2α)

41-Nb-92

Cross Section

-9.370 To 9999. %



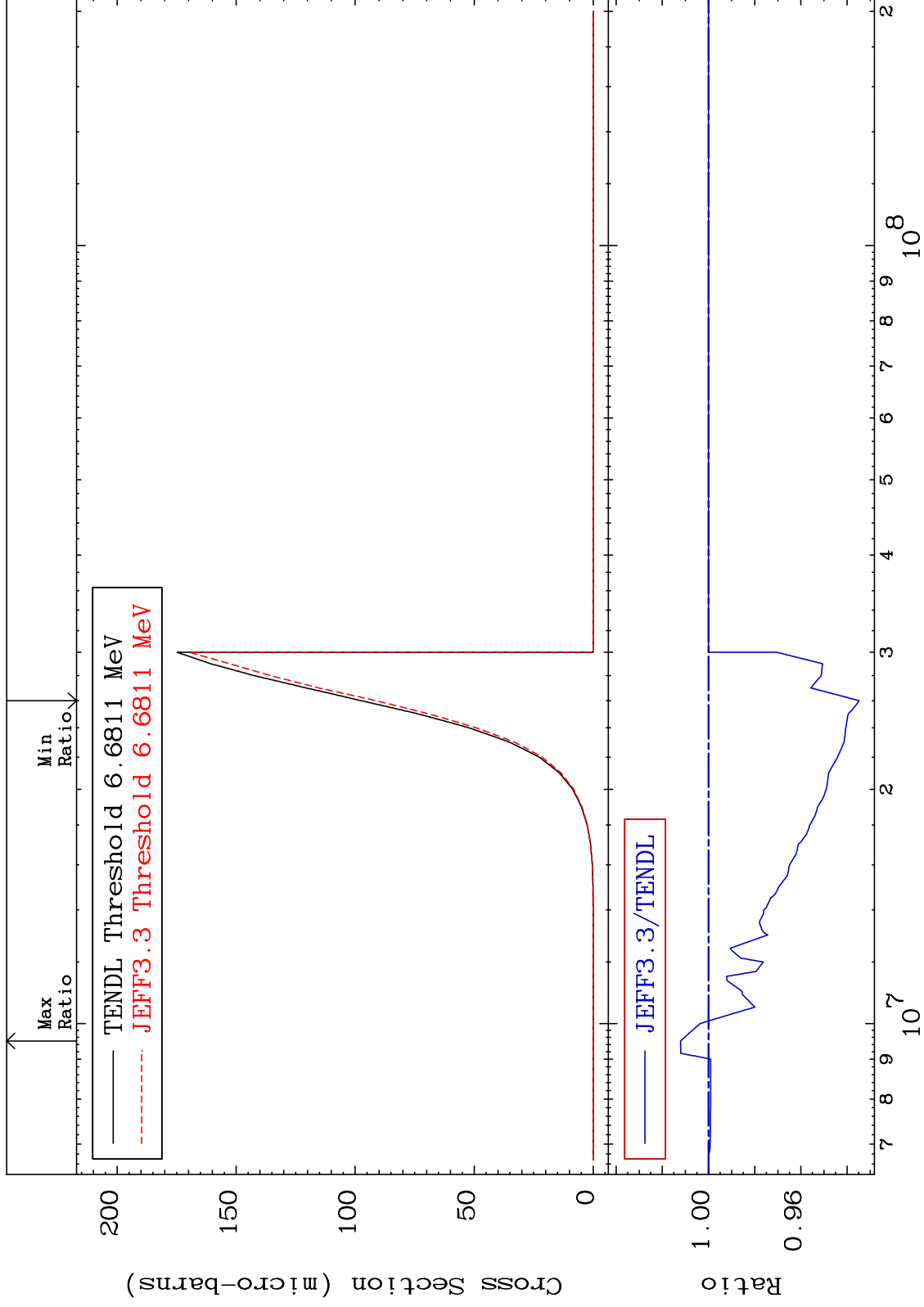
MAT 4122

(n,2p)

41-Nb-92

Cross Section

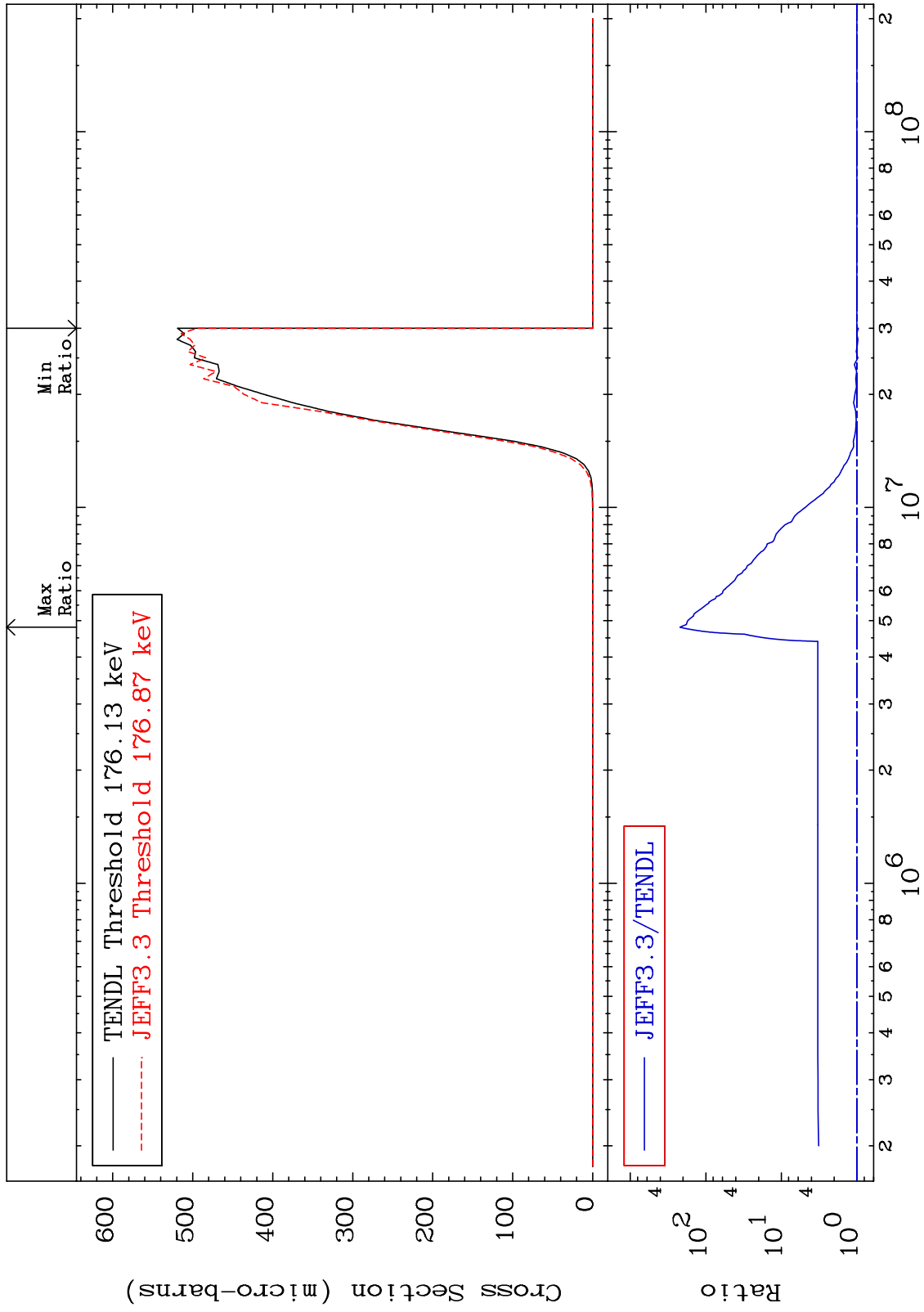
-6.520 To 1.207 %

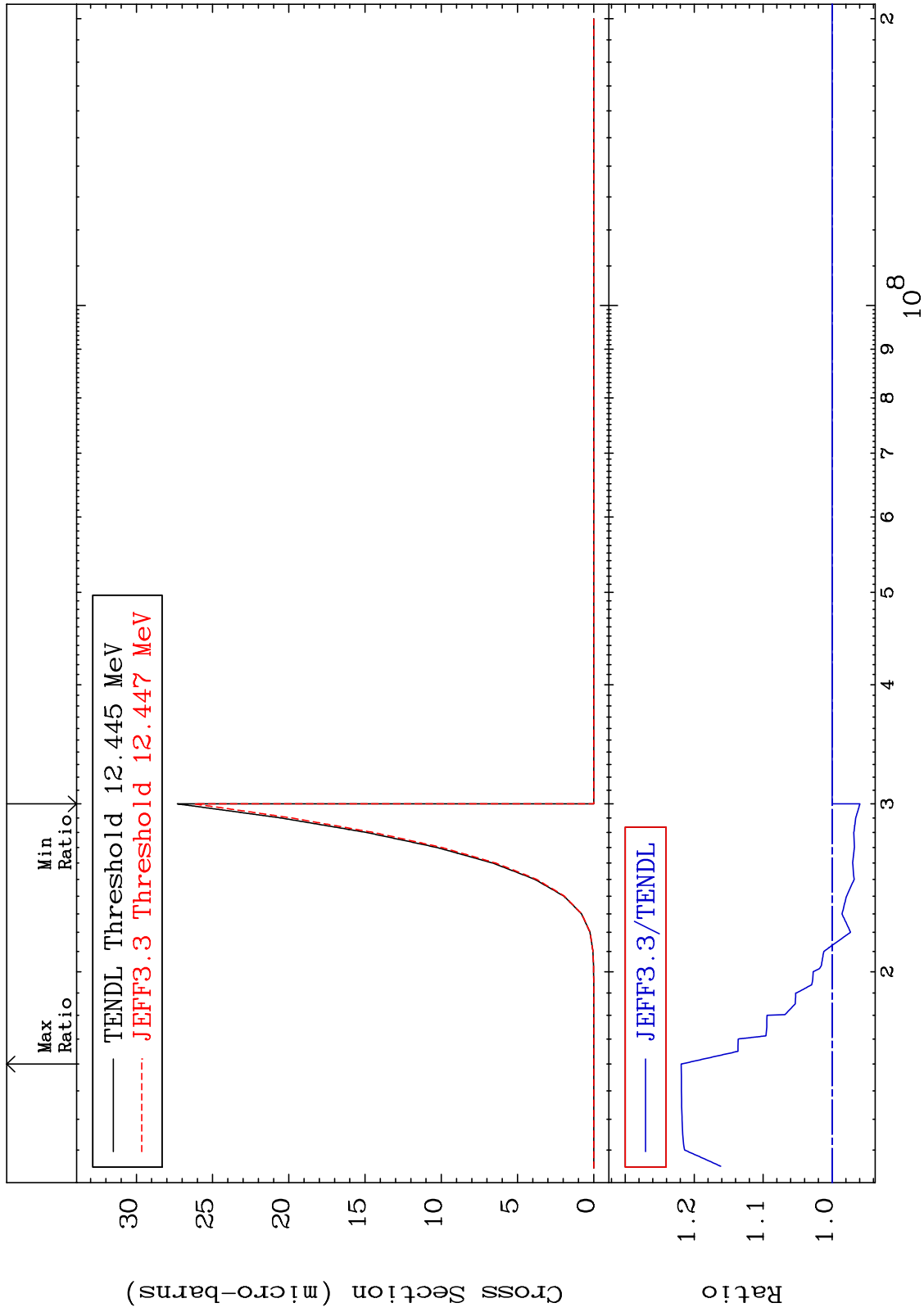


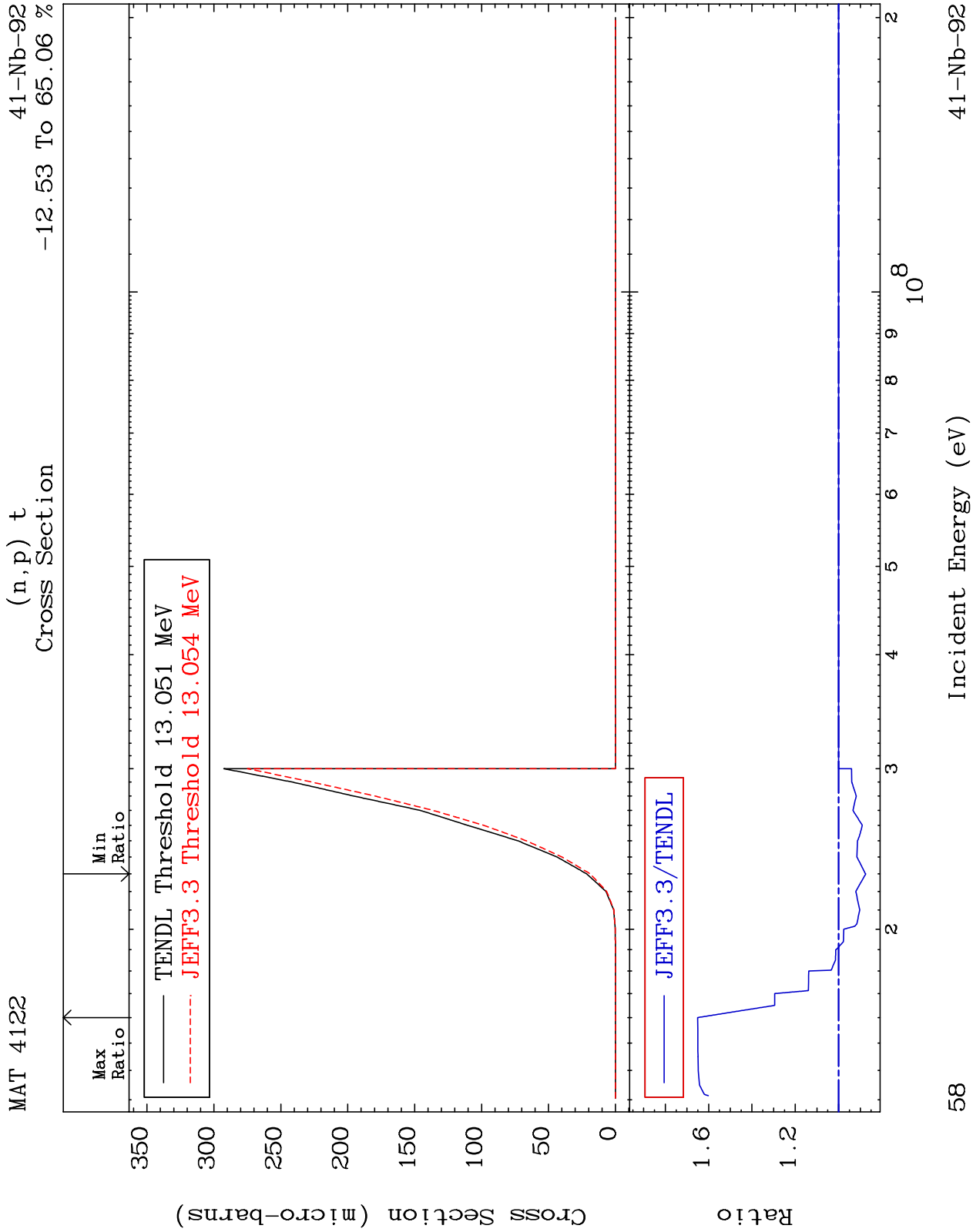
55

Incident Energy (eV)

41-Nb-92







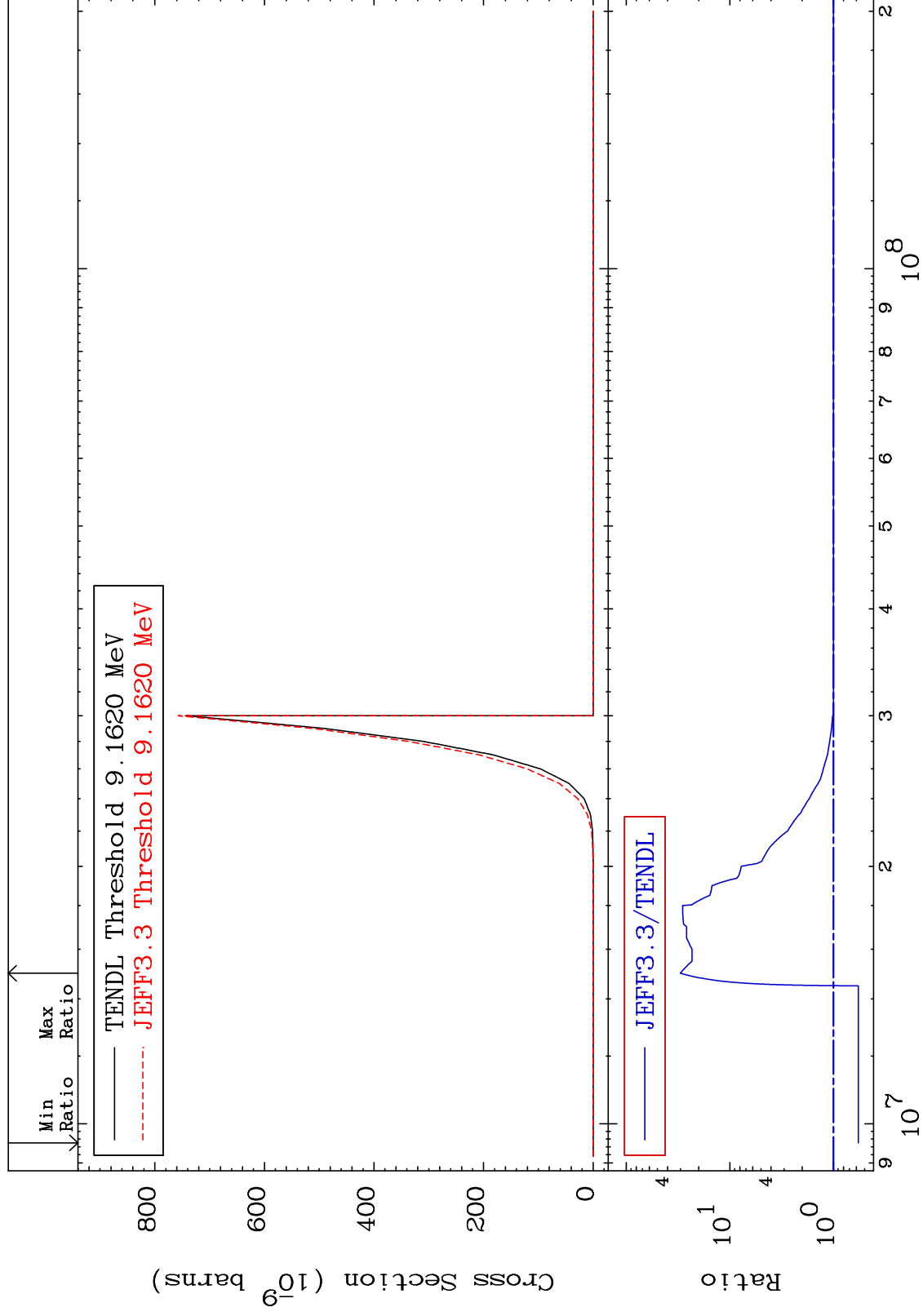
MAT 4122

(n,d) α

41-Nb-92

Cross Section

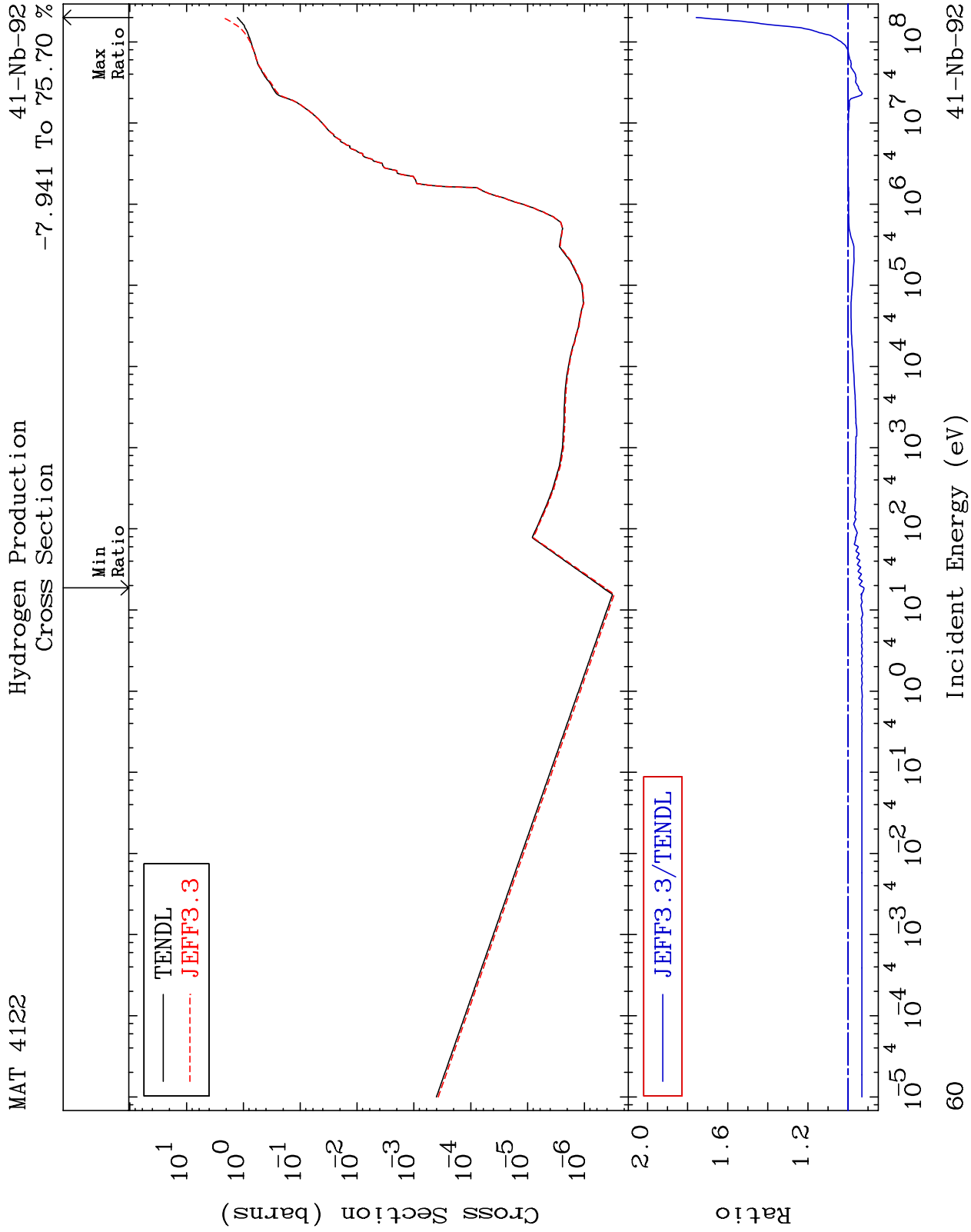
-42.54 To 2909. %



59

Incident Energy (eV)

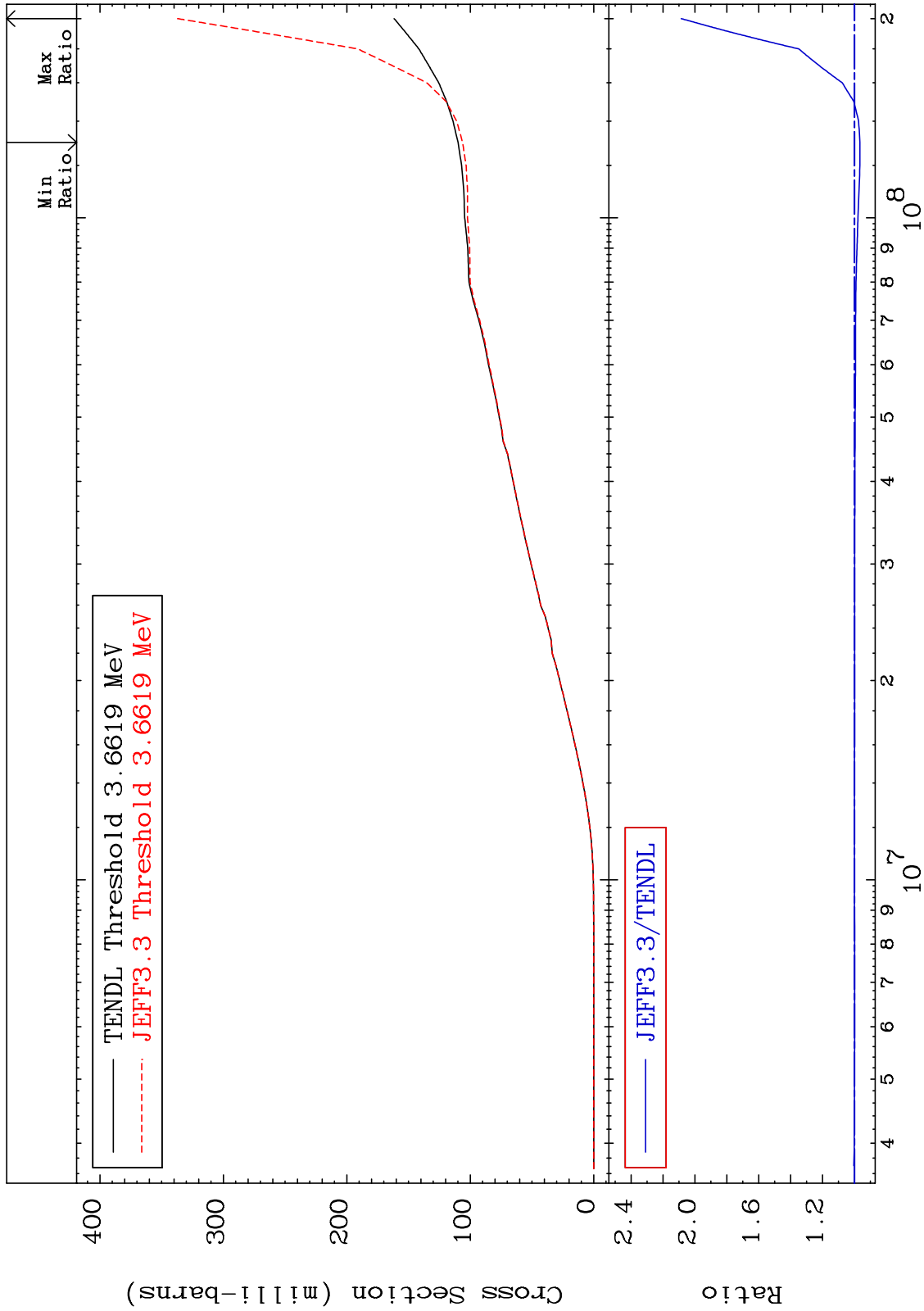
41-Nb-92



MAT 4122

Deuterium Production
Cross Section

41-Nb-92
-3.388 To 108.6 %



61

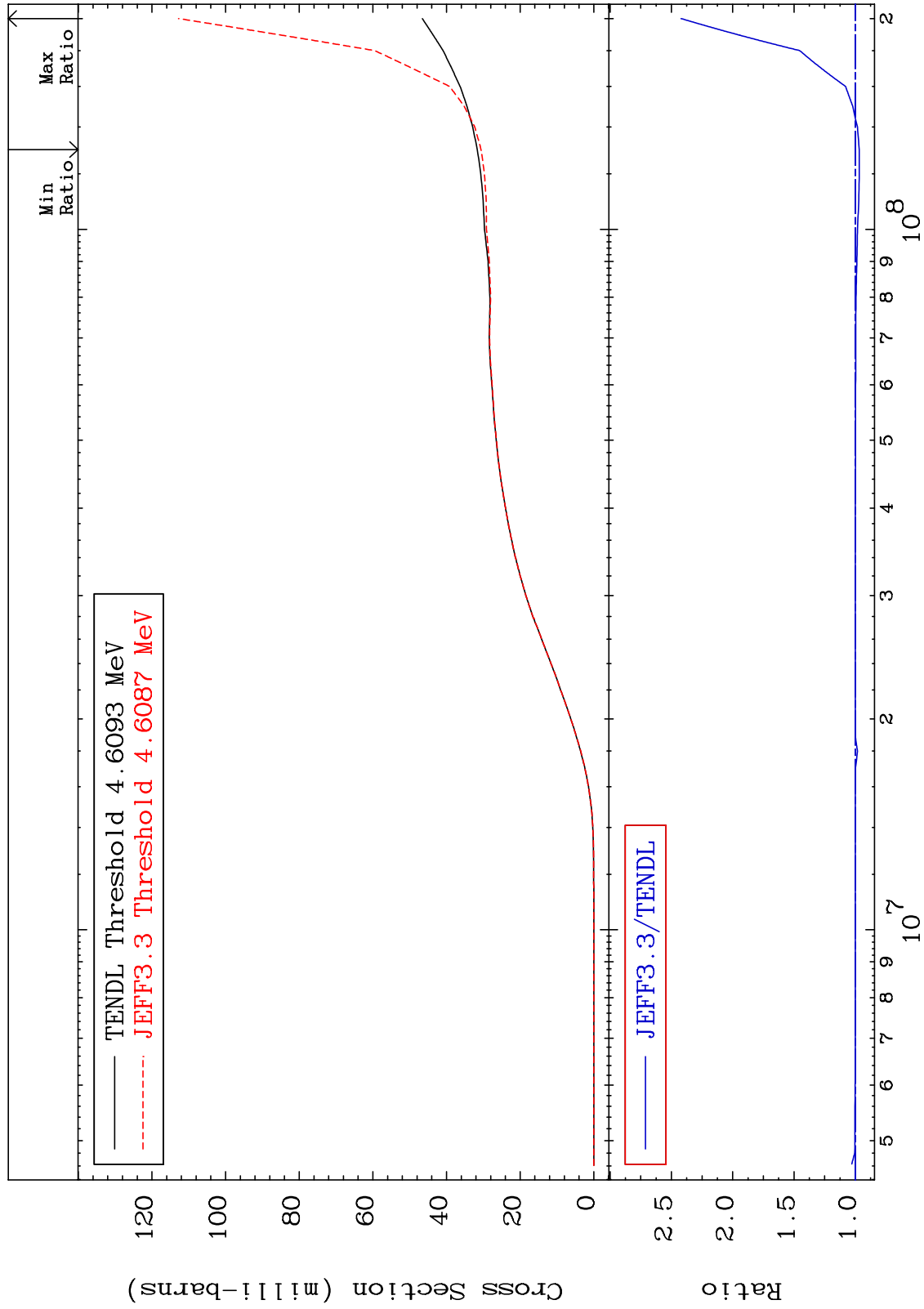
Incident Energy (eV)

41-Nb-92

MAT 4122

Tritium Production
Cross Section

41-Nb-92
-3.225 To 142.1 %



62

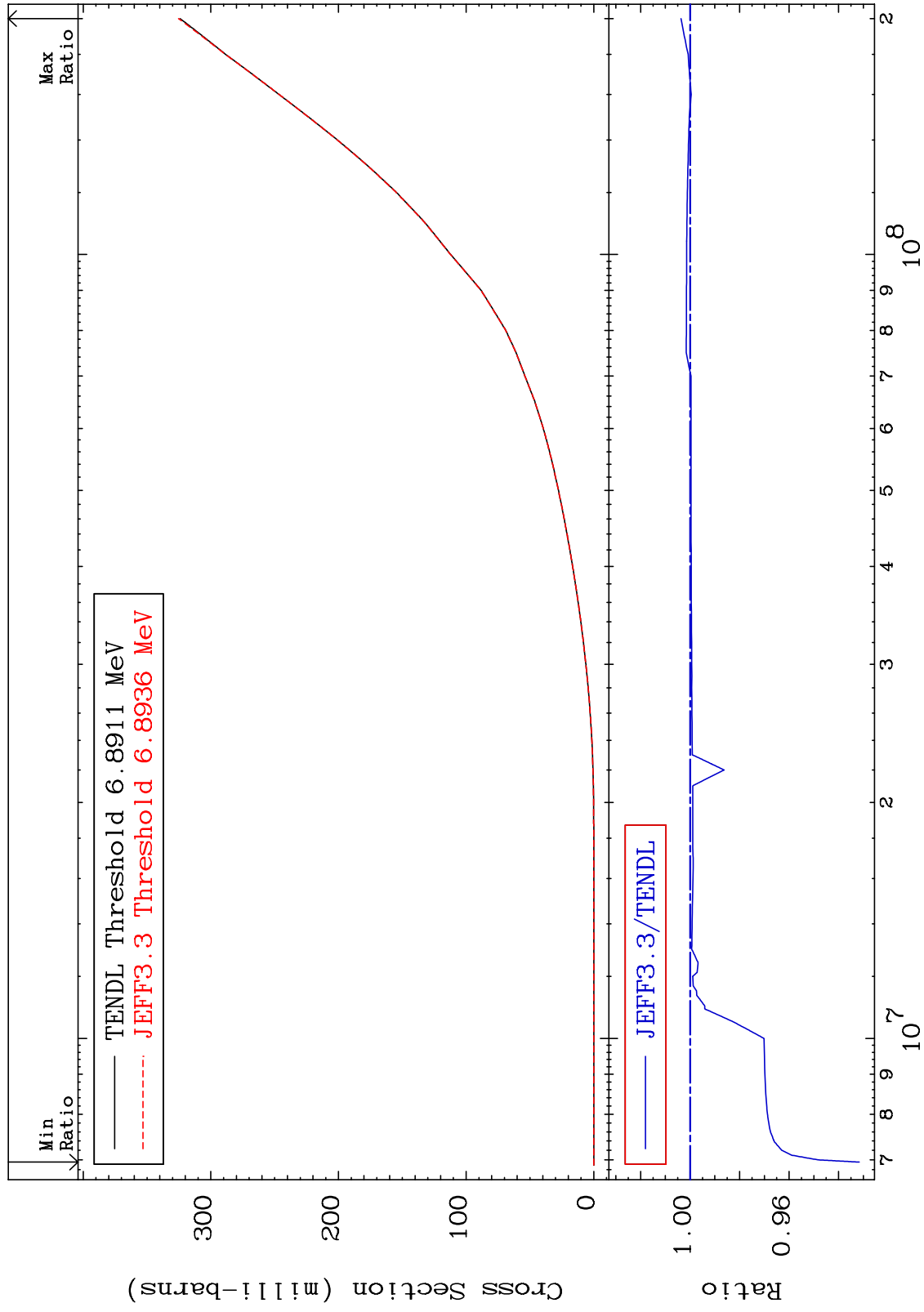
Incident Energy (eV)

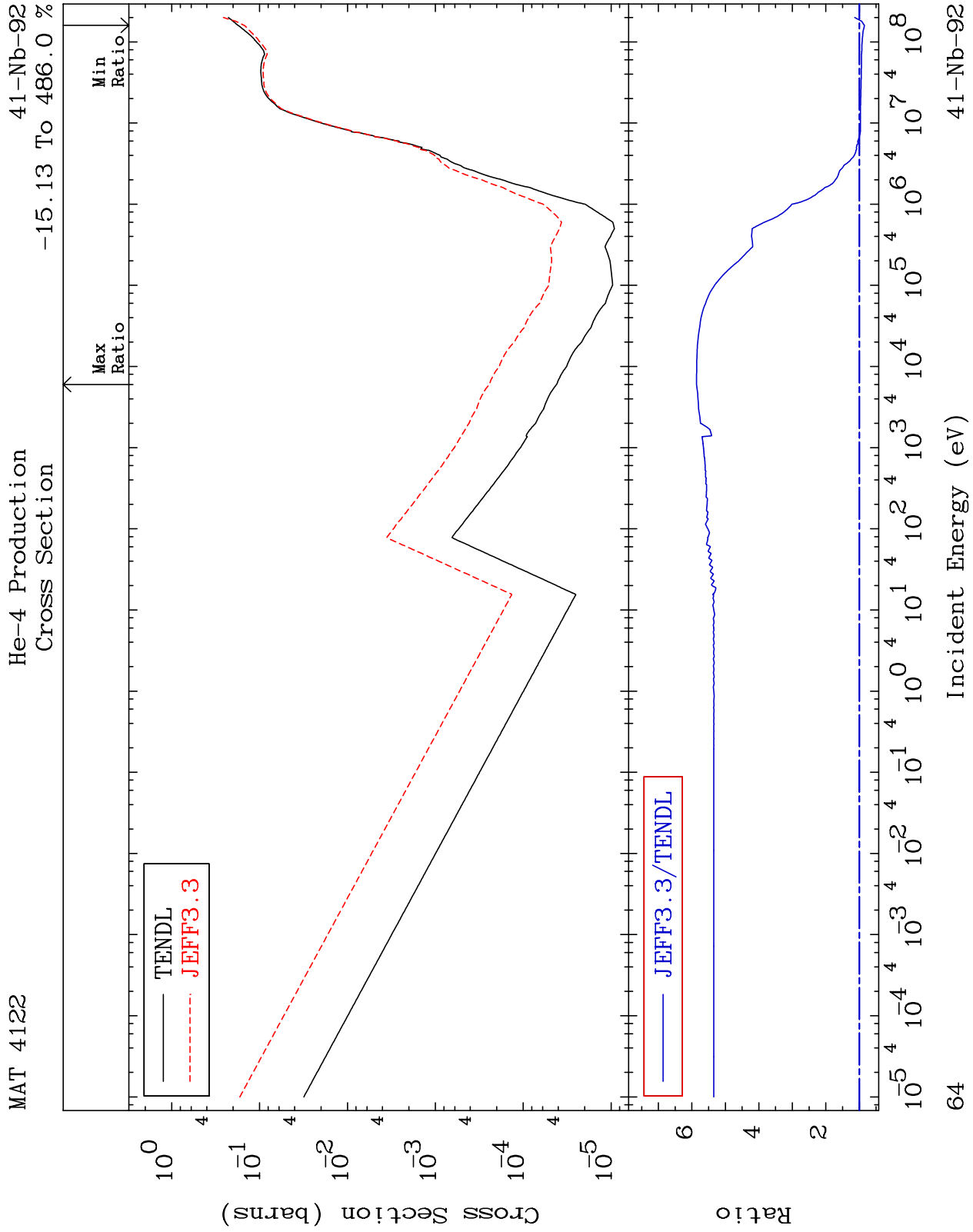
41-Nb-92

MAT 4122

He-3 Production
Cross Section

41-Nb-92
-6.837 To 0.366 %





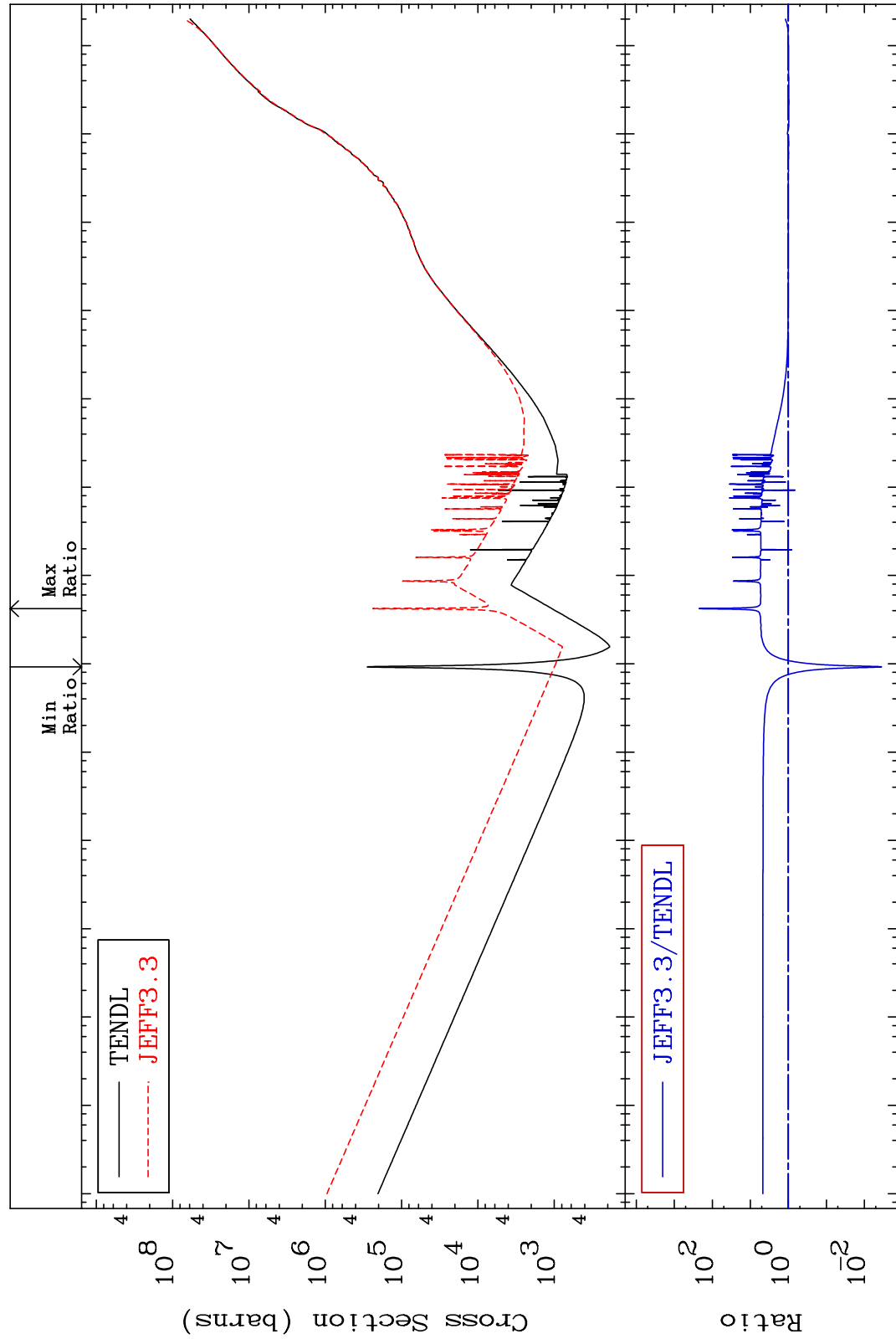
MAT 4122

Kerma total (eV-barns)

41-Nb-92

-99.65 To 9999. %

Cross Section



65

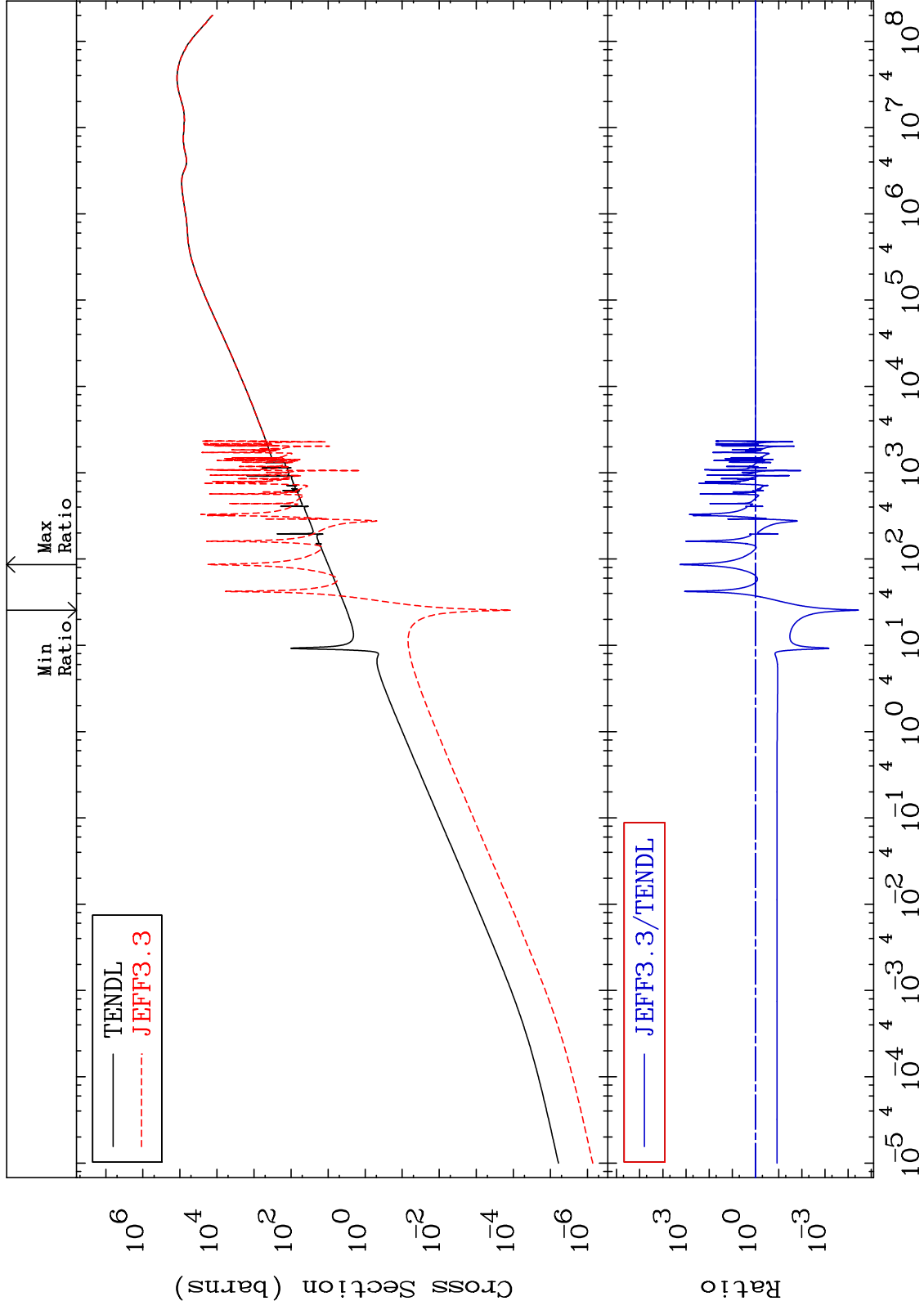
Incident Energy (eV)

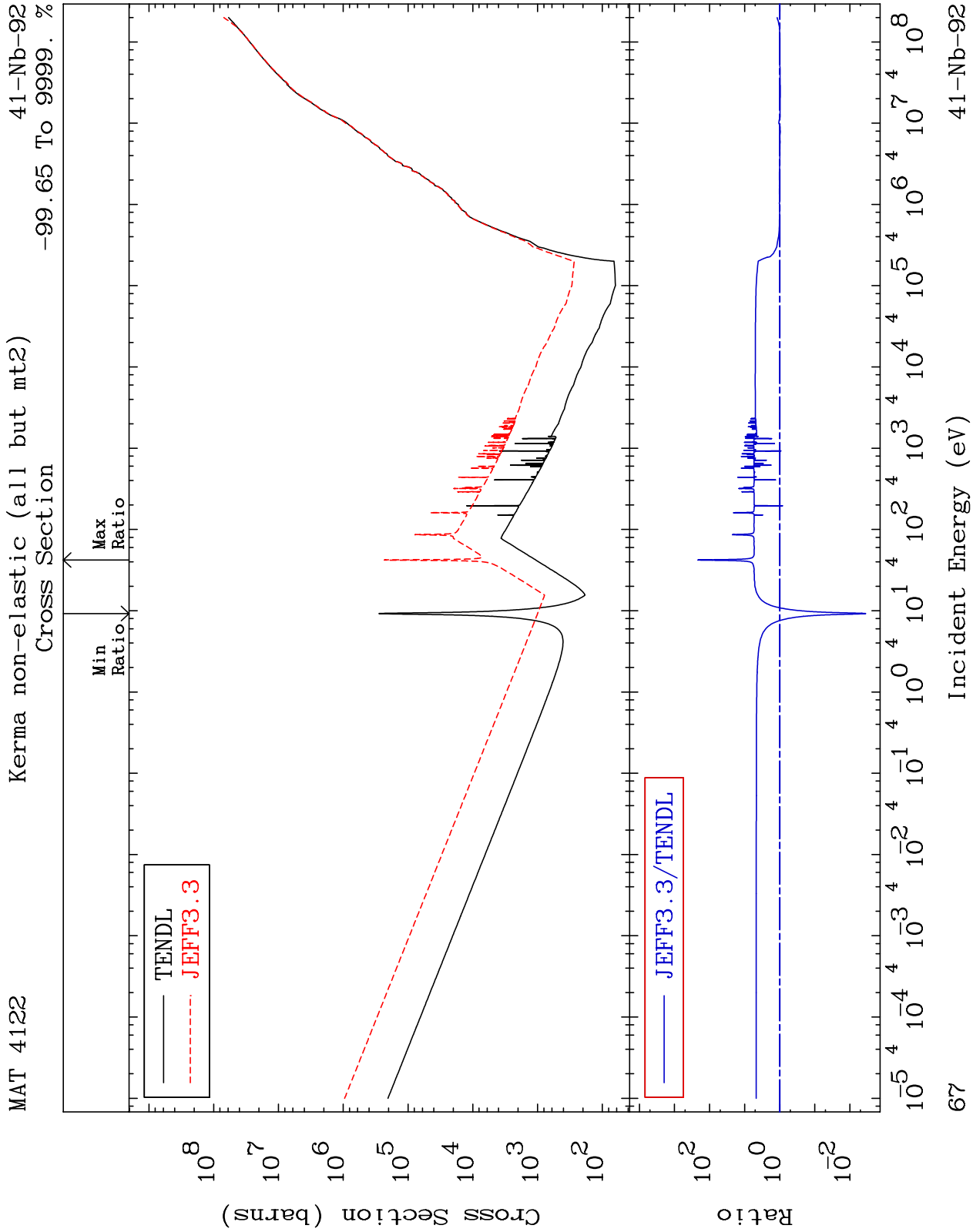
41-Nb-92

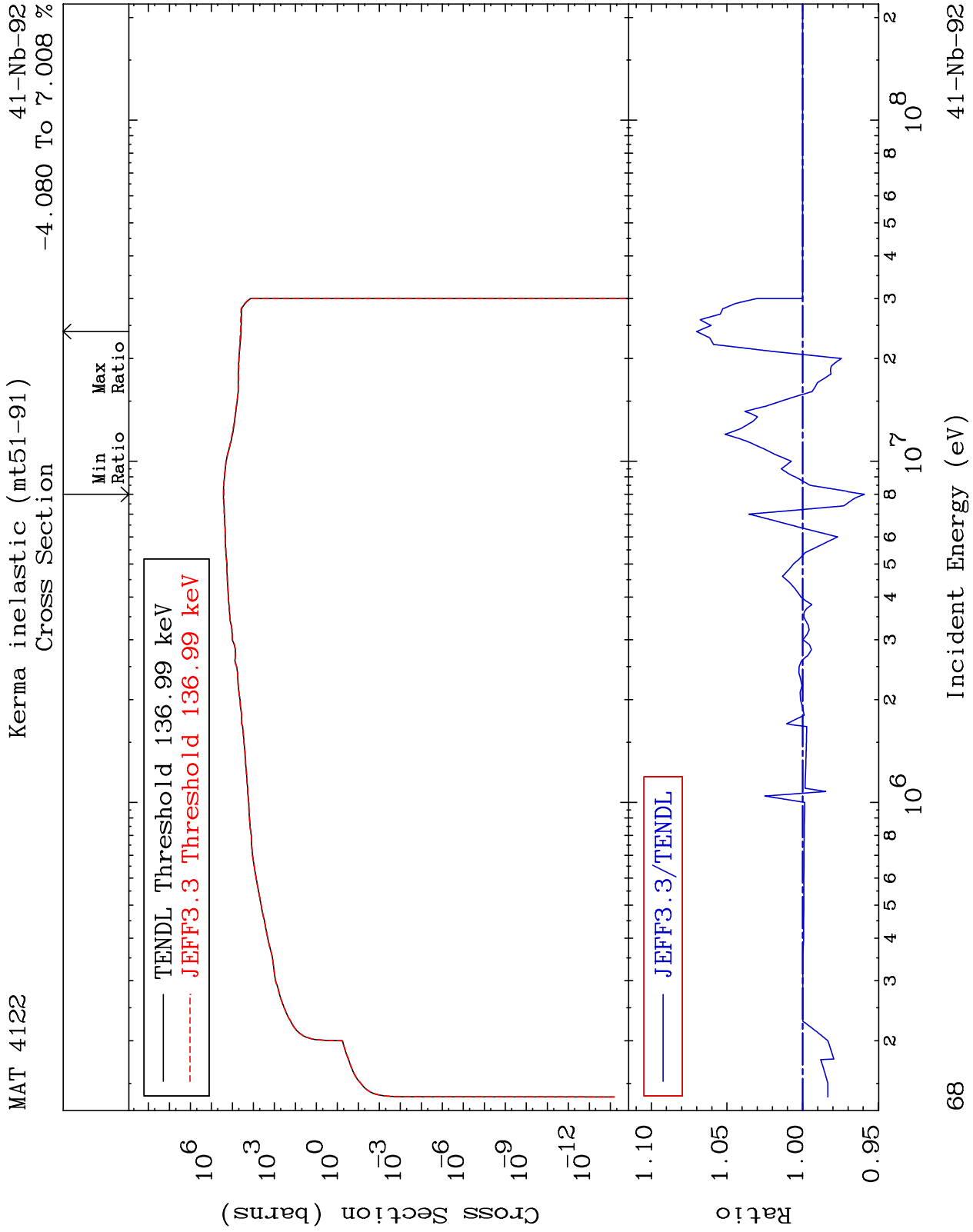
MAT 4122

Kerma elastic
Cross Section

41-Nb-92
-100.0 To 9999. %



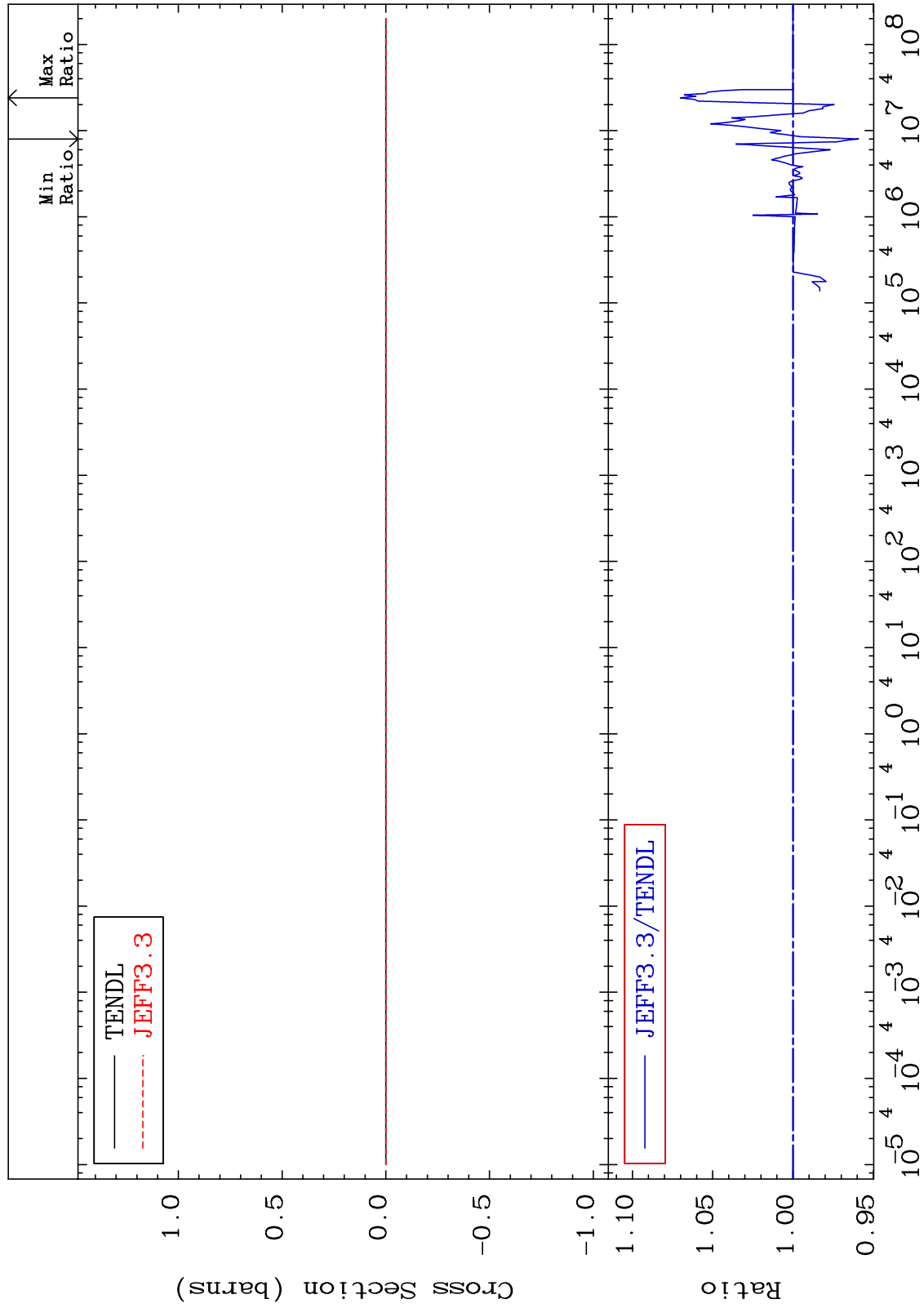




MAT 4122

Kerma fission (mt18 or mt19-20-21-38)
Cross Section

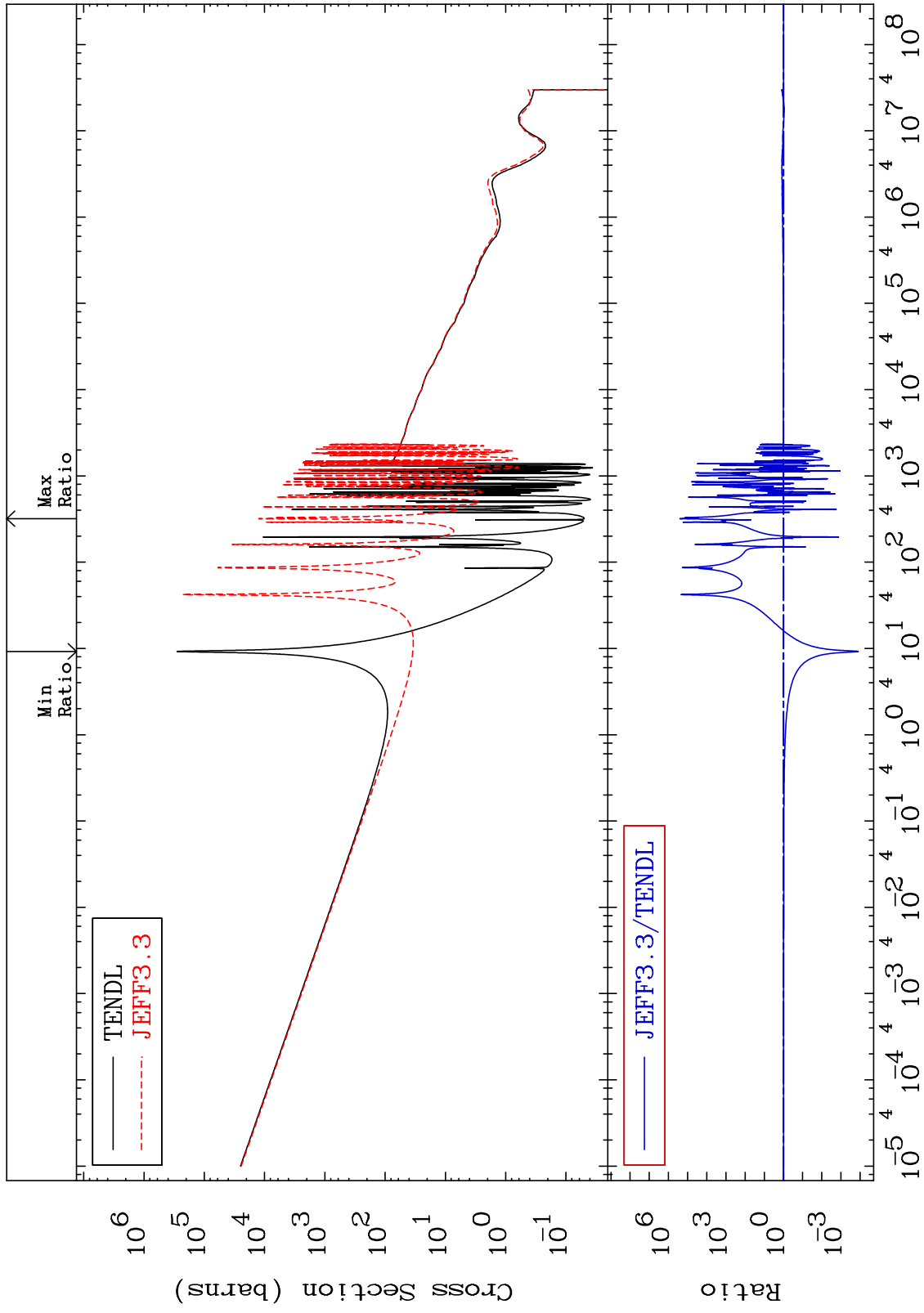
41-Nb-92
-4.080 To 7.008 %



MAT 4122

Kerma capture (mt102)
Cross Section

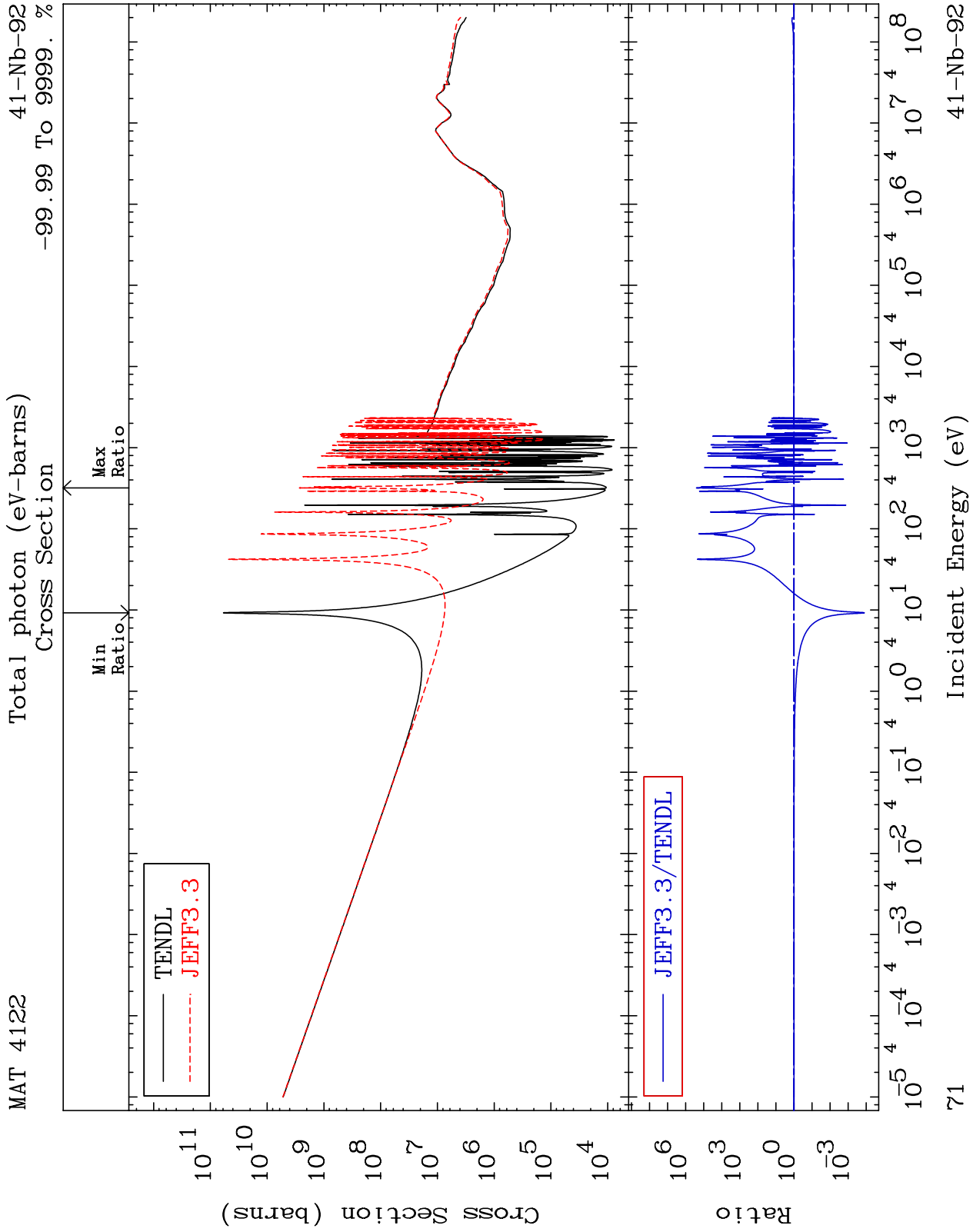
41-Nb-92
-99.99 To 9999. %

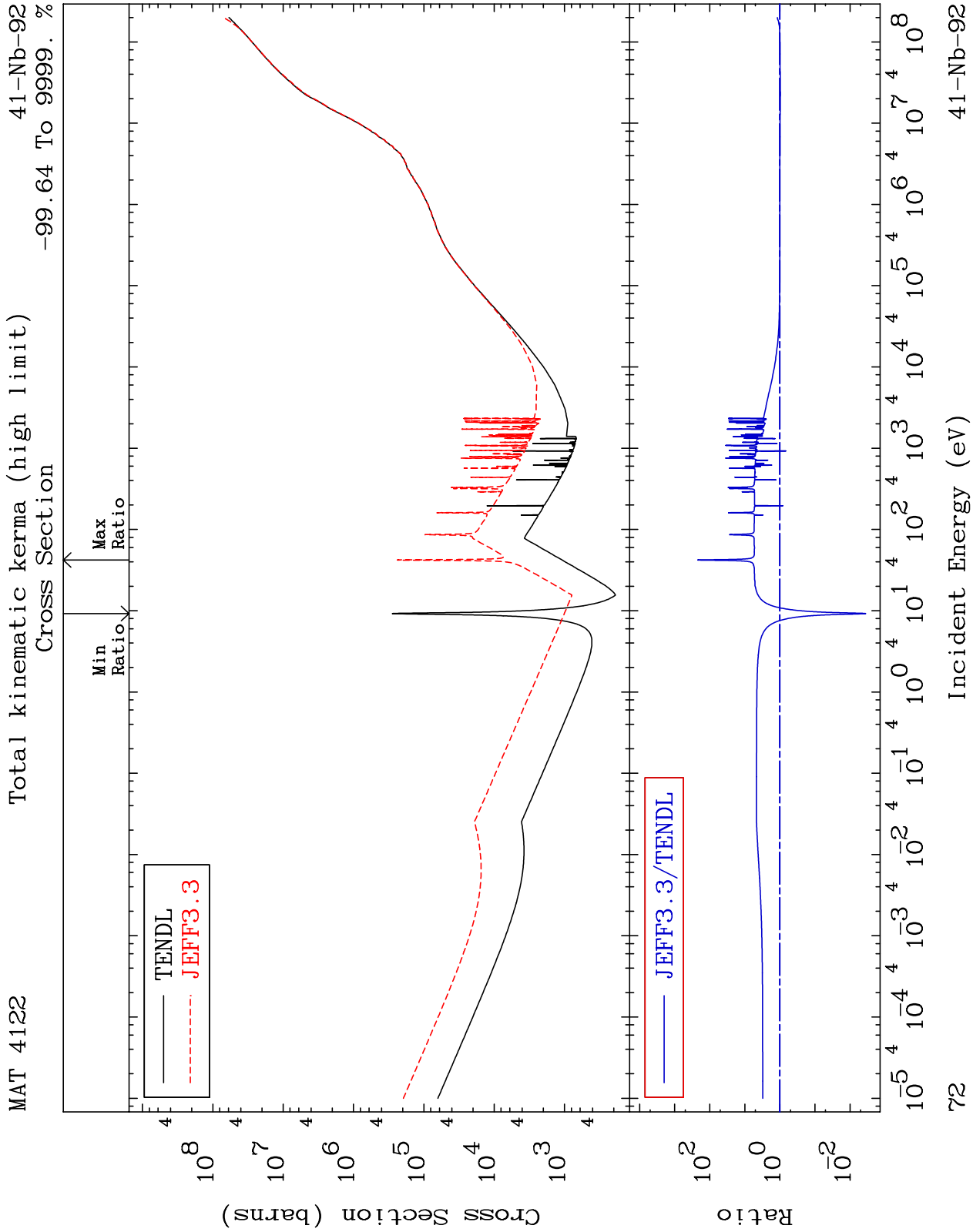


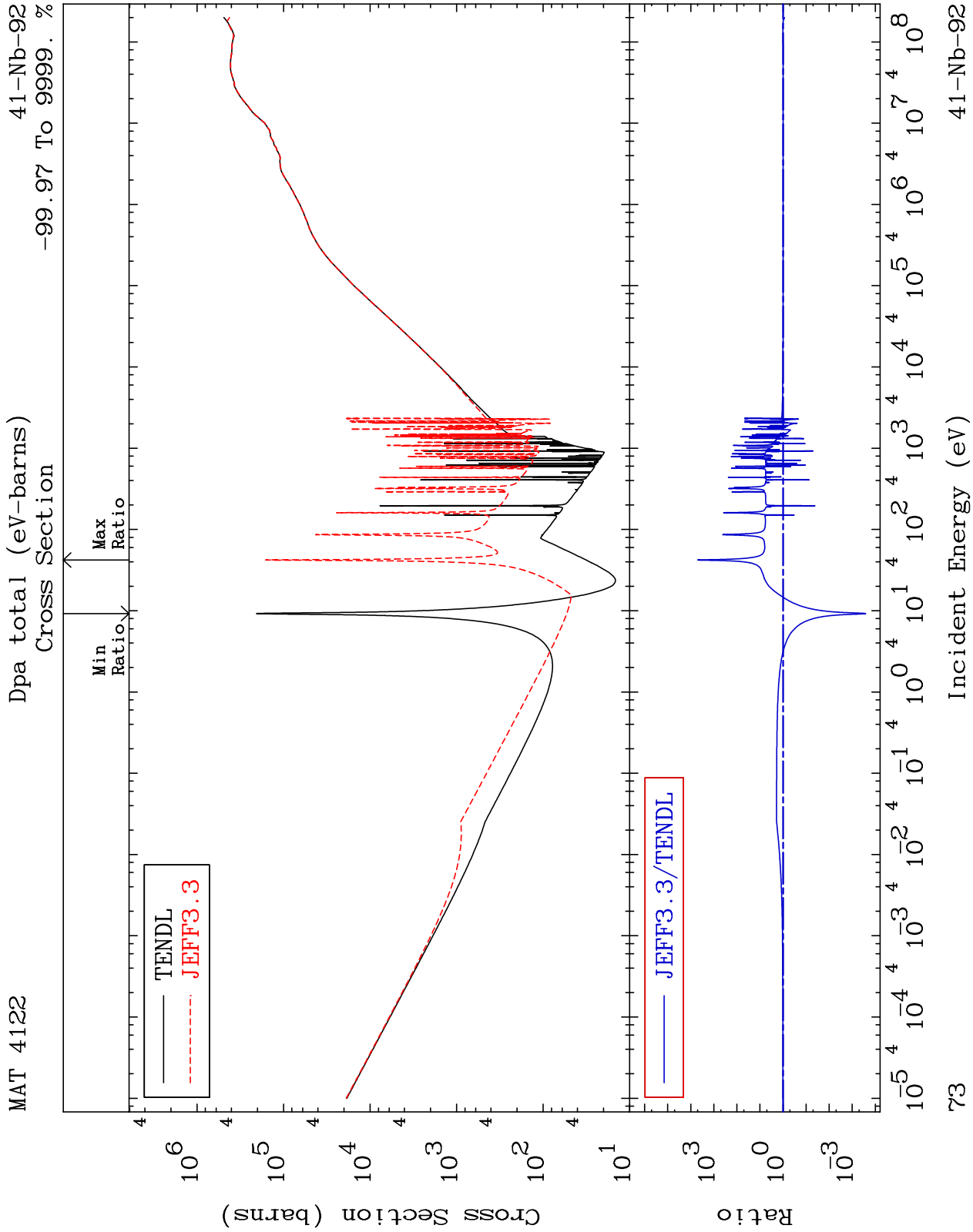
70

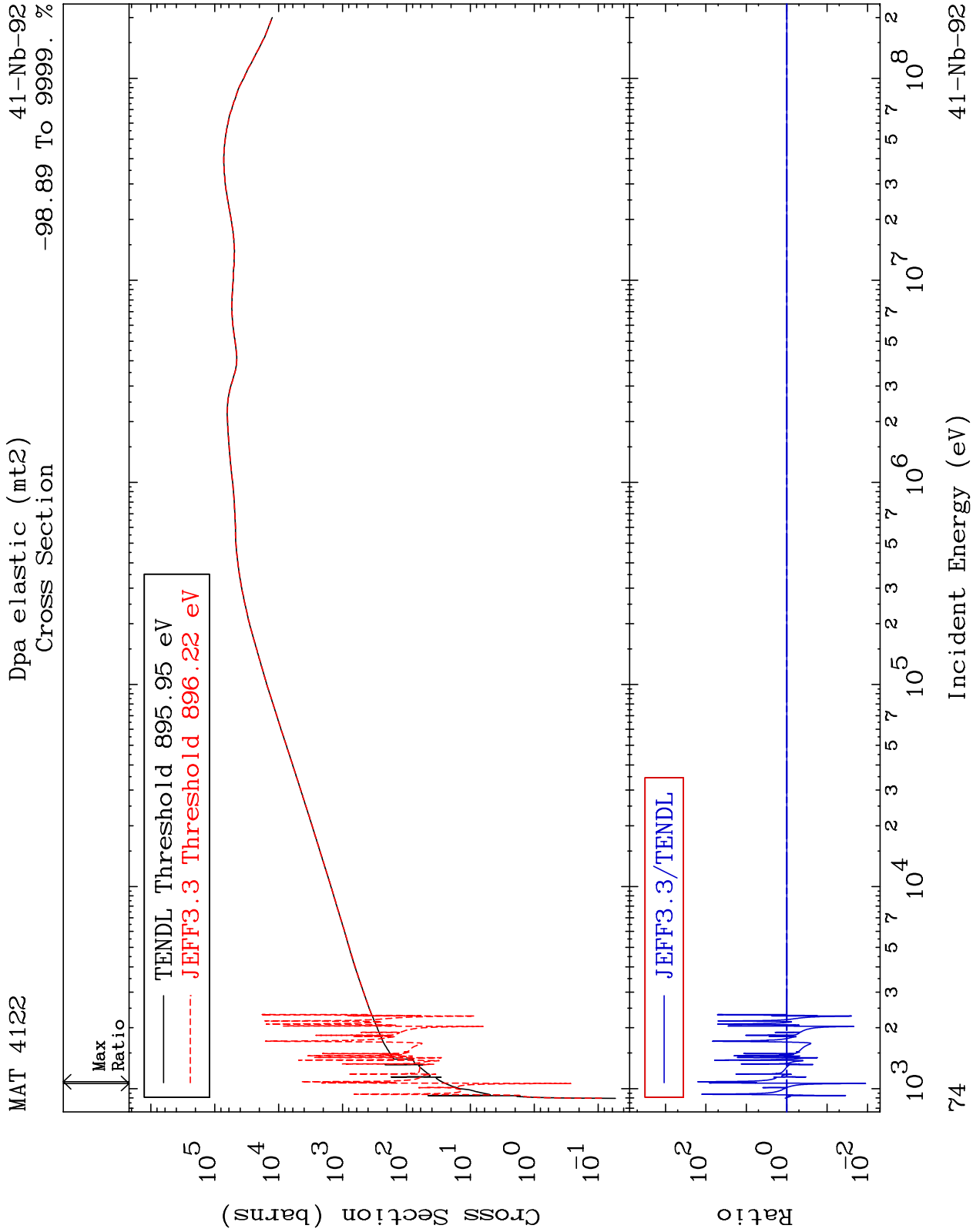
Incident Energy (eV)

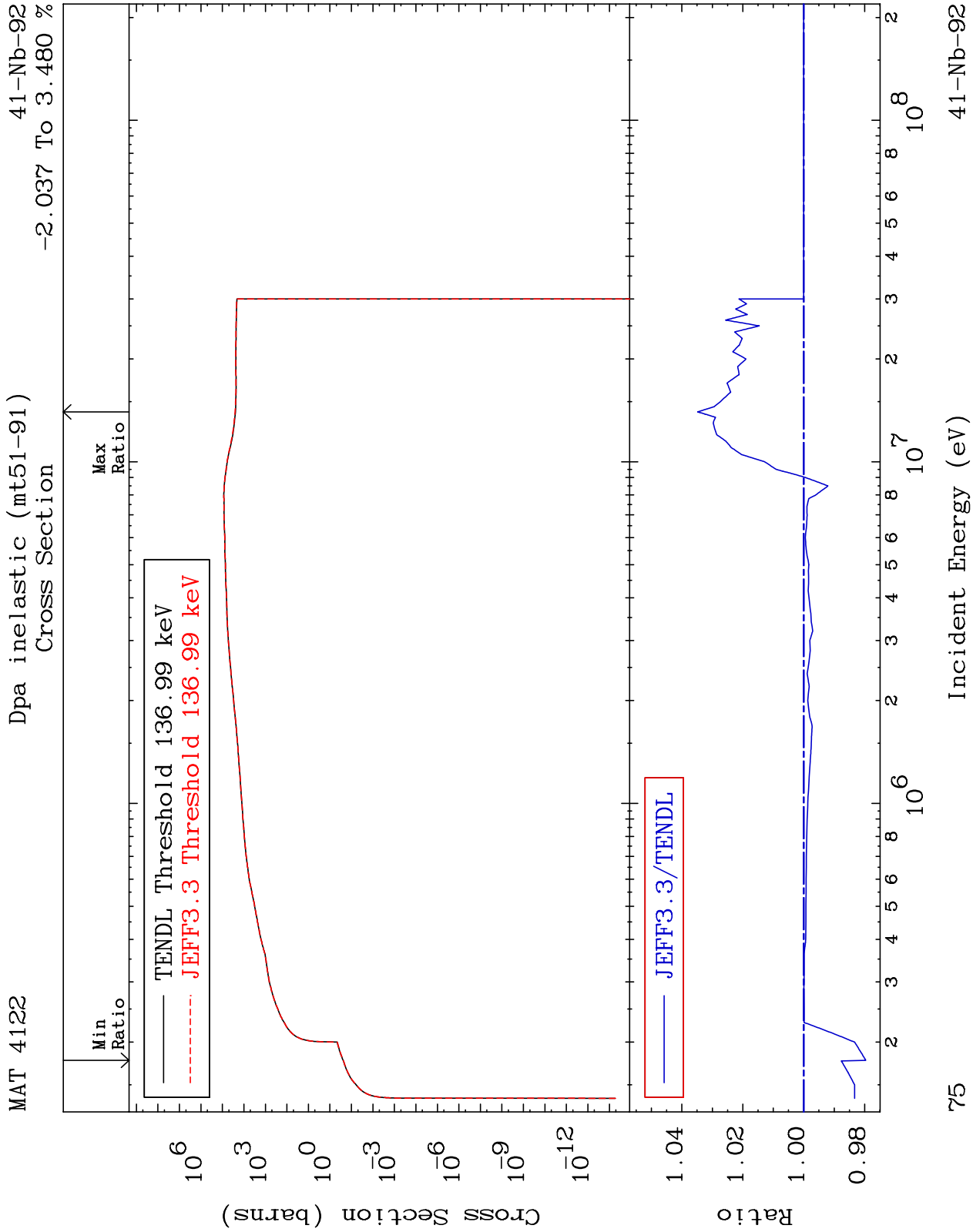
41-Nb-92

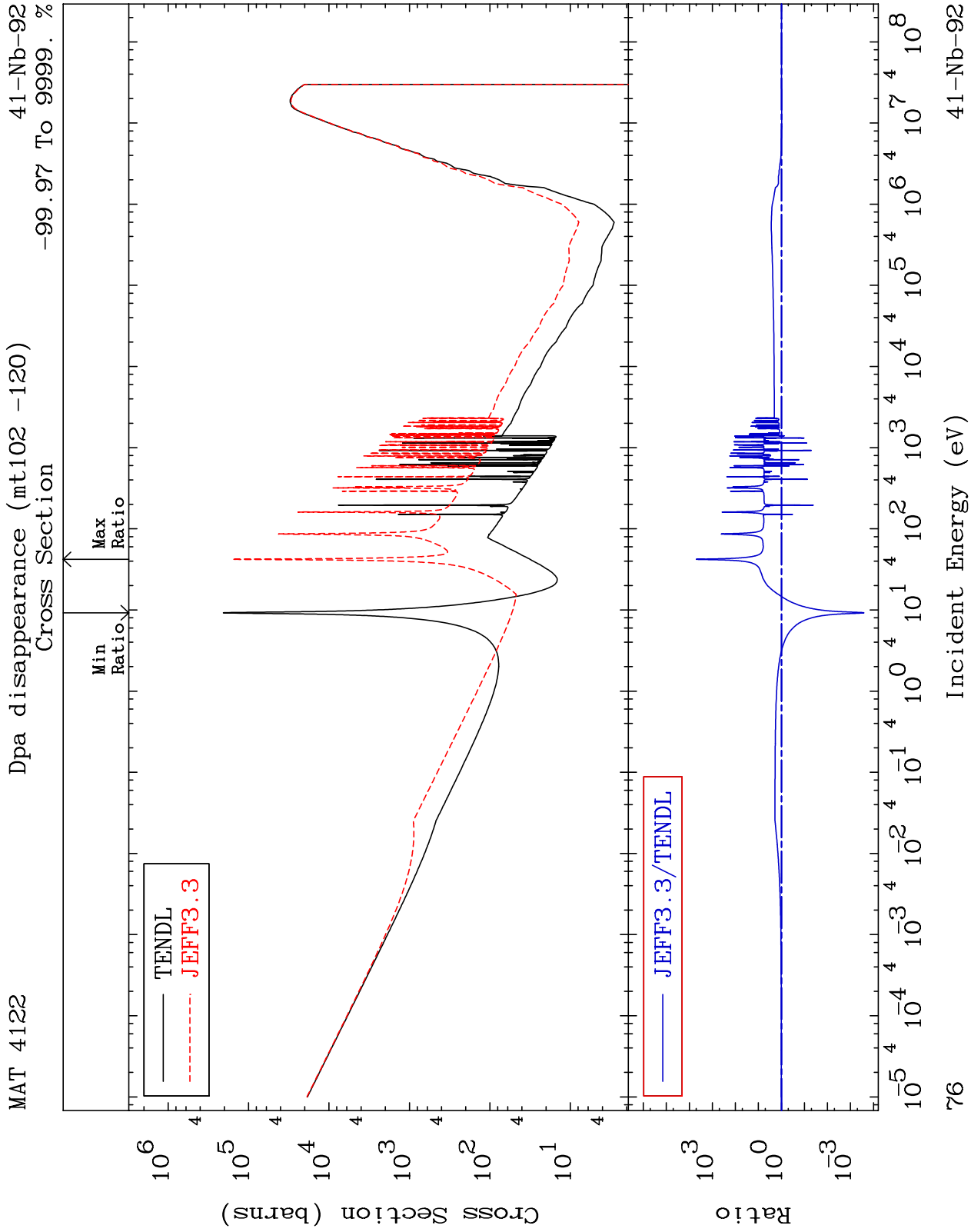










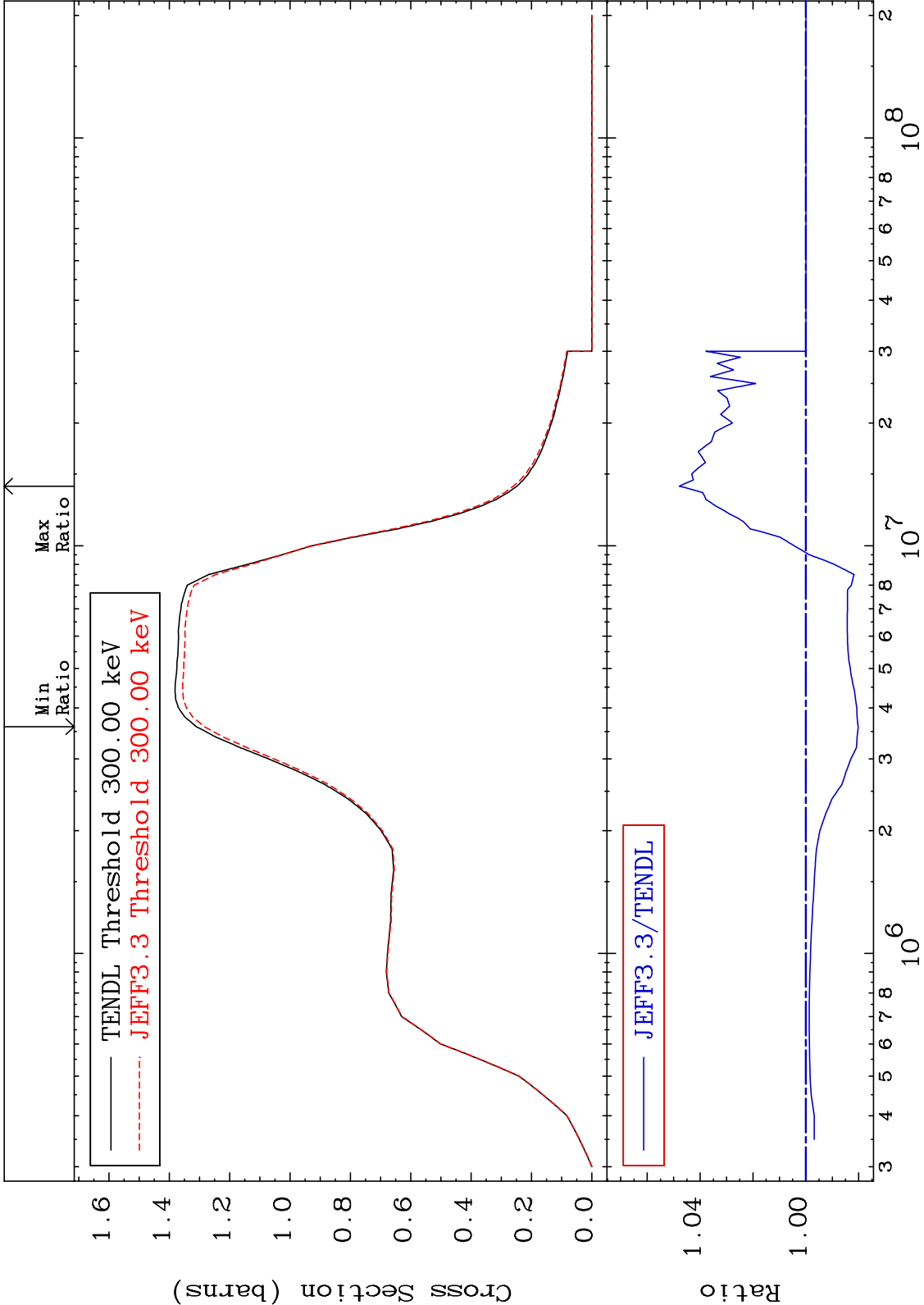


MAT 4122

41-Nb-92

Inelastic: 41-Nb-92g

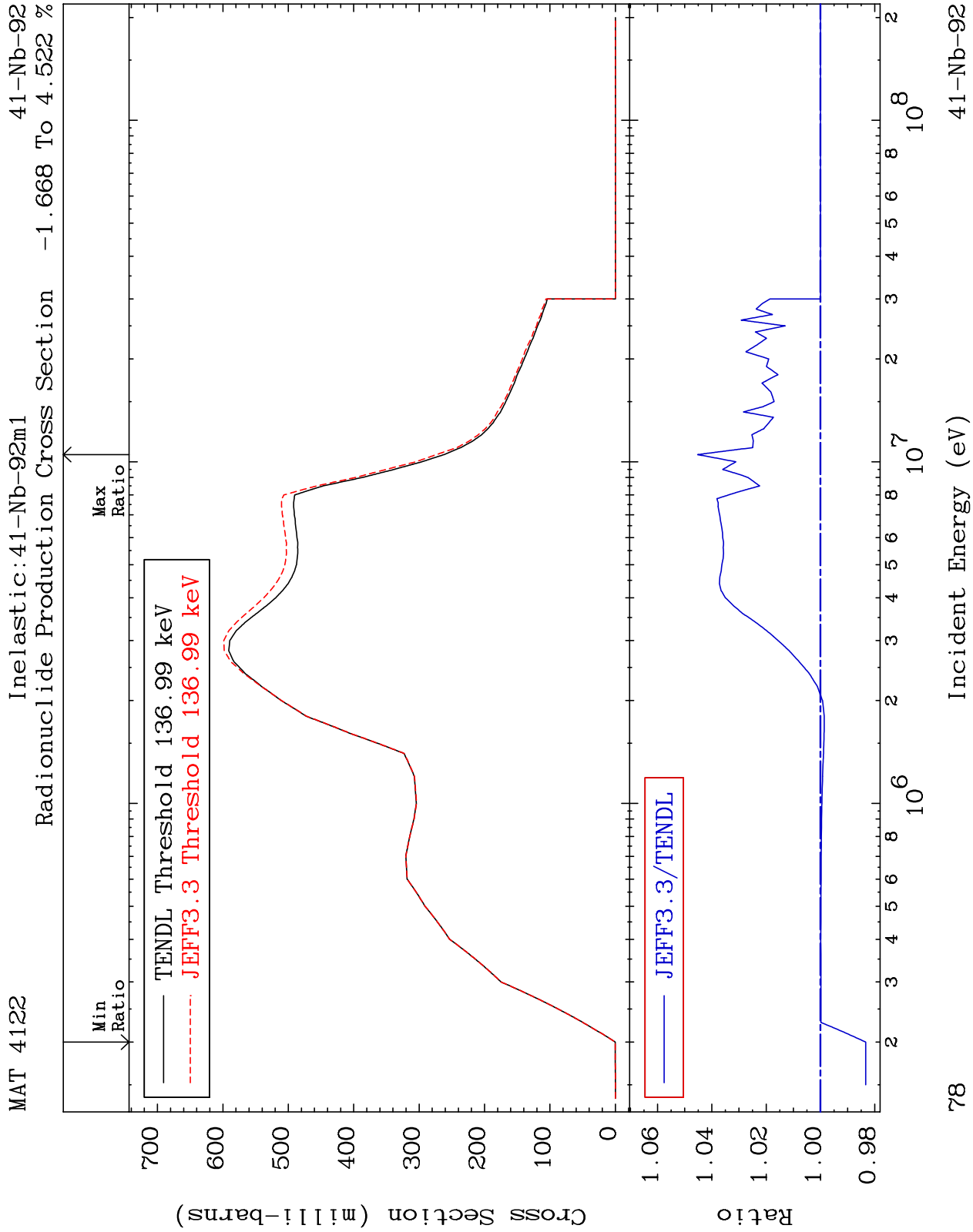
Radionuclide Production Cross Section -1.981 To 4.784 %



Incident Energy (eV)

41-Nb-92

77

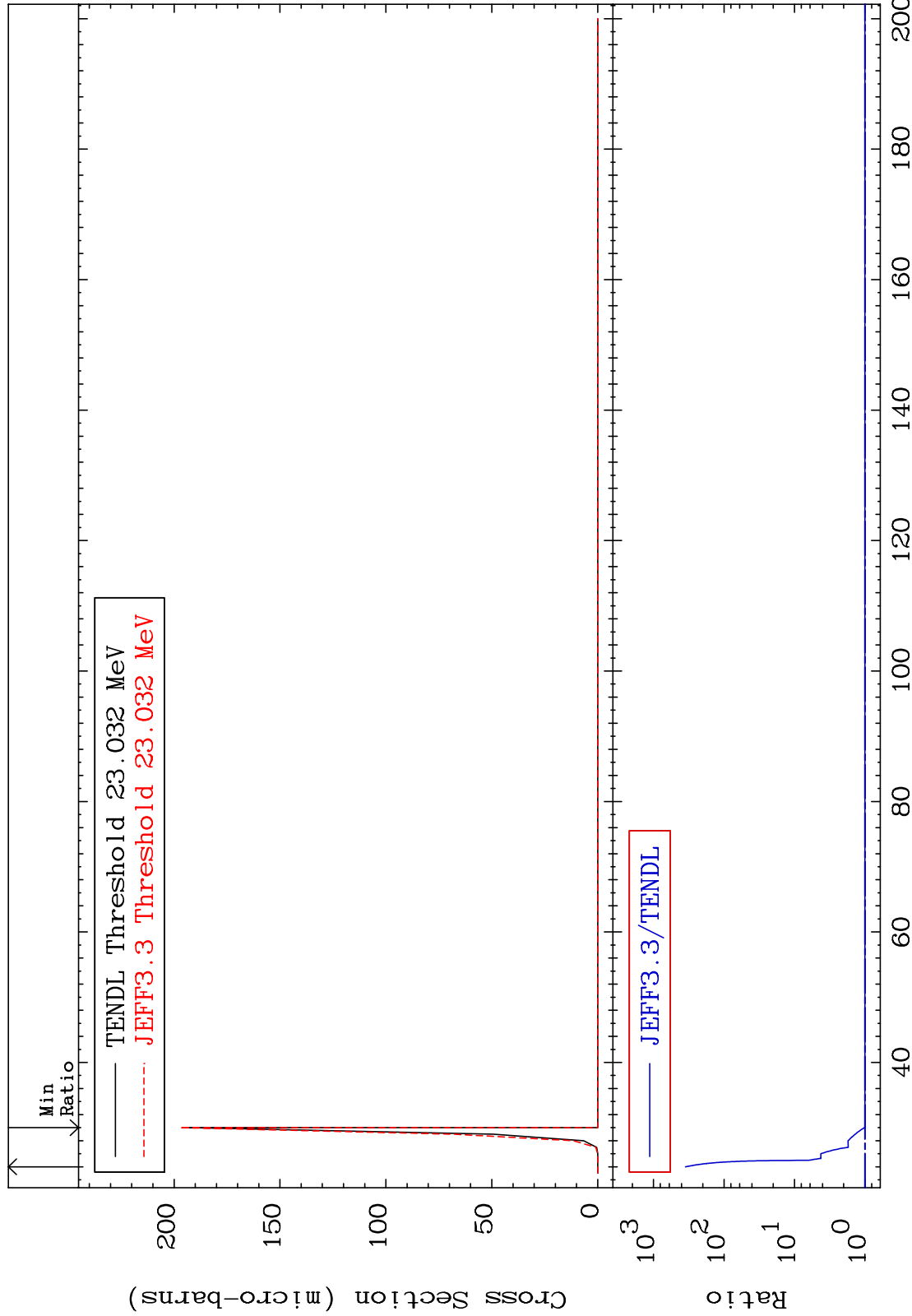


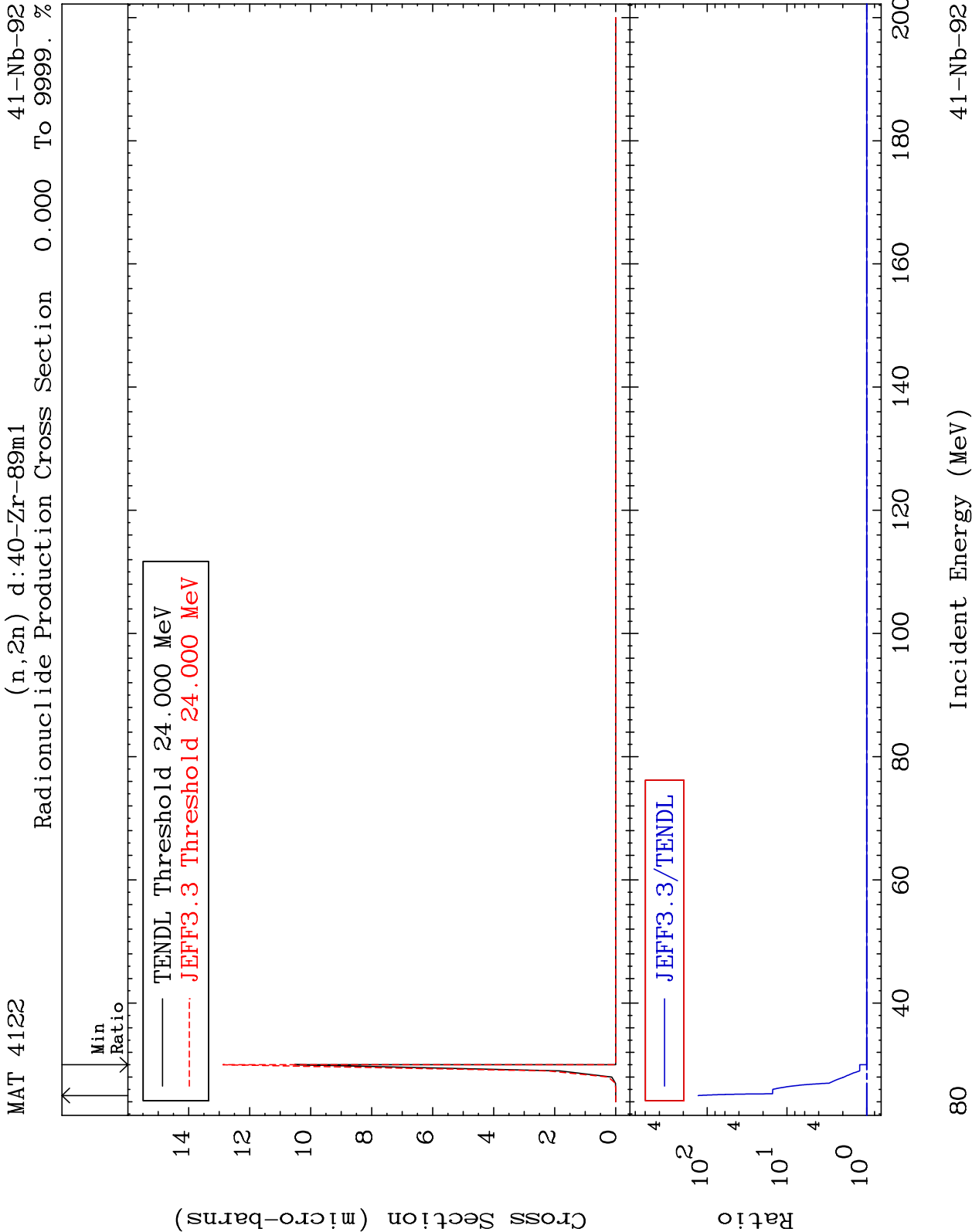
MAT 4122

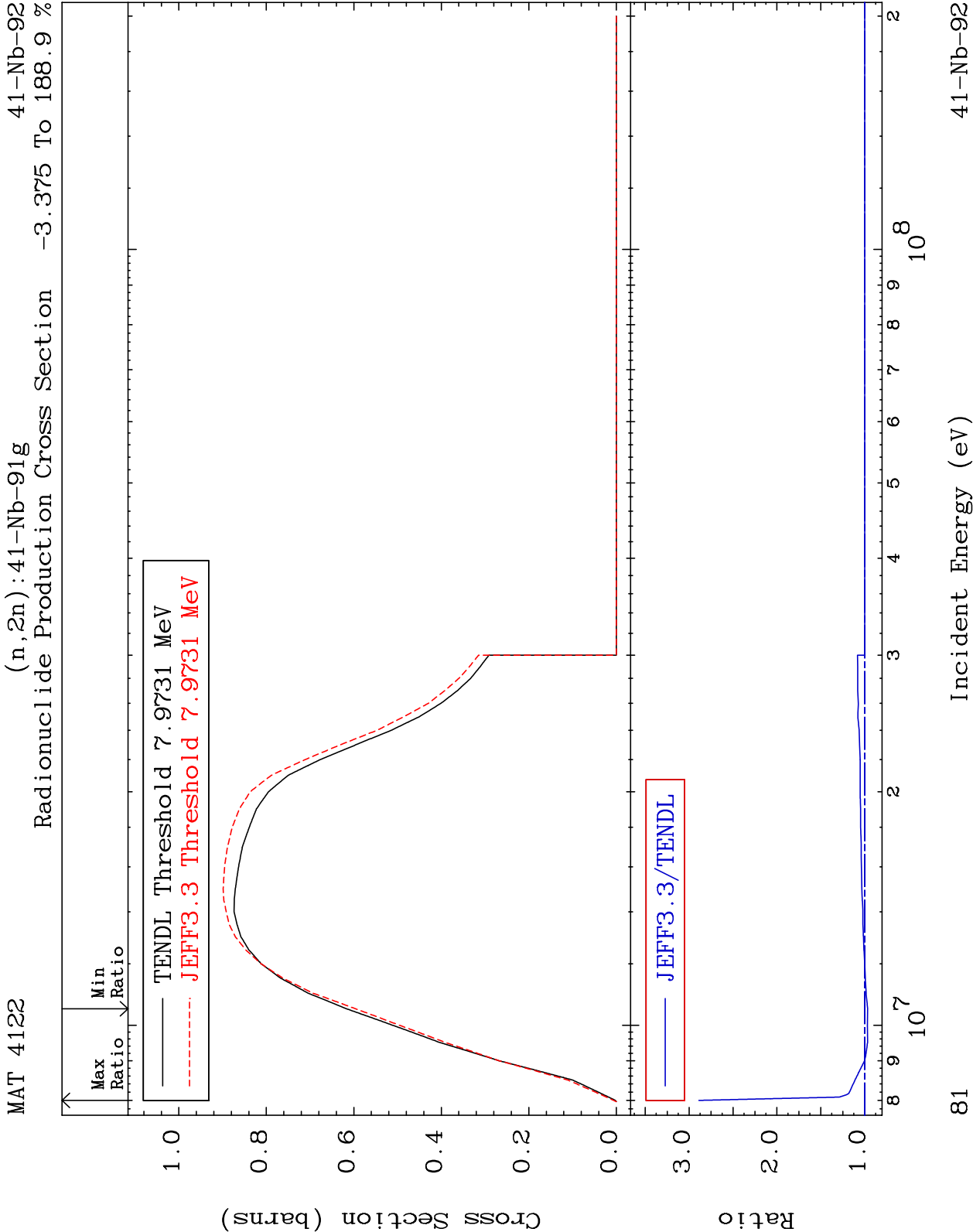
(n,2n) d:40-Zr-89g

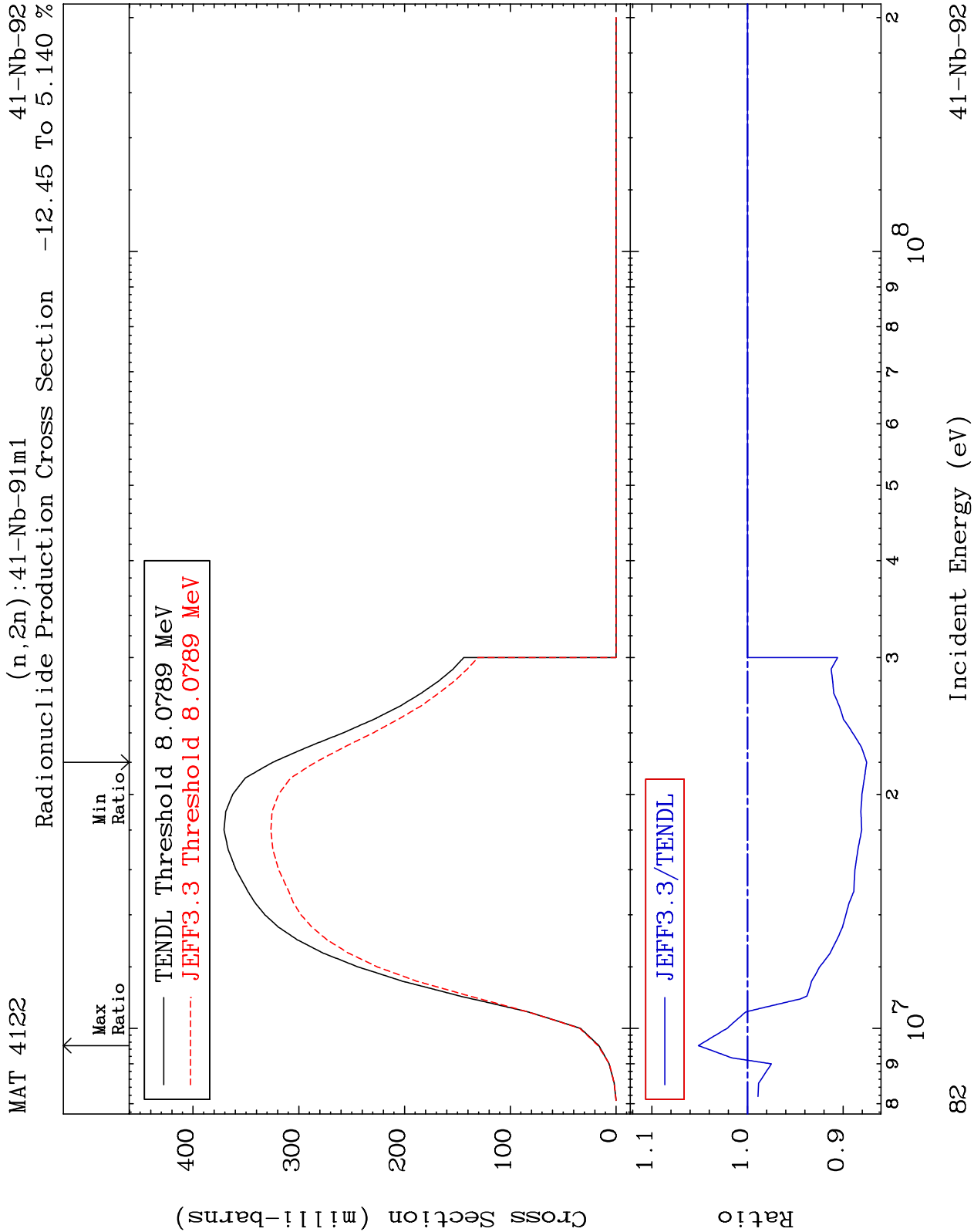
41-Nb-92

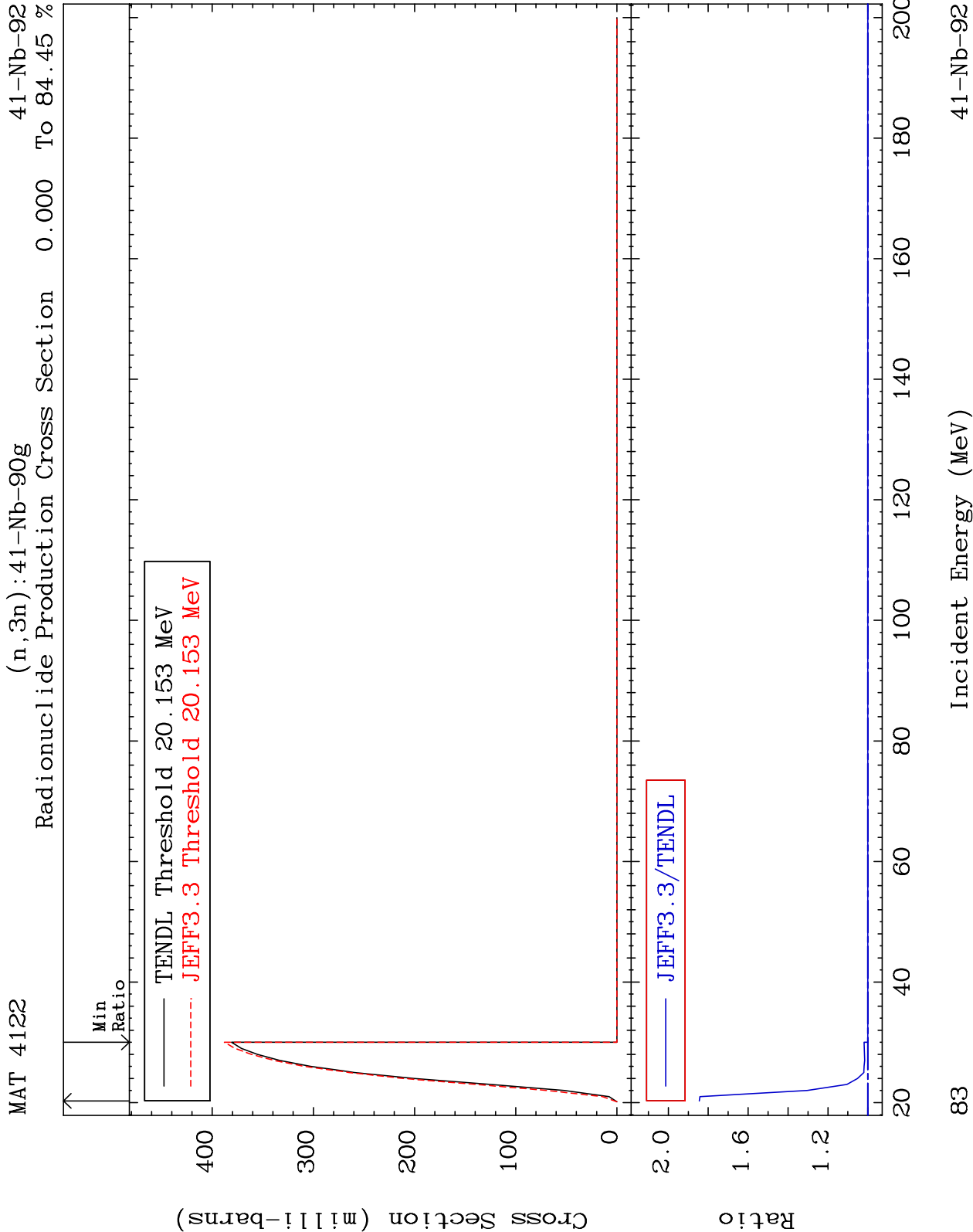
Radionuclide Production Cross Section 0.000 To 9999. %











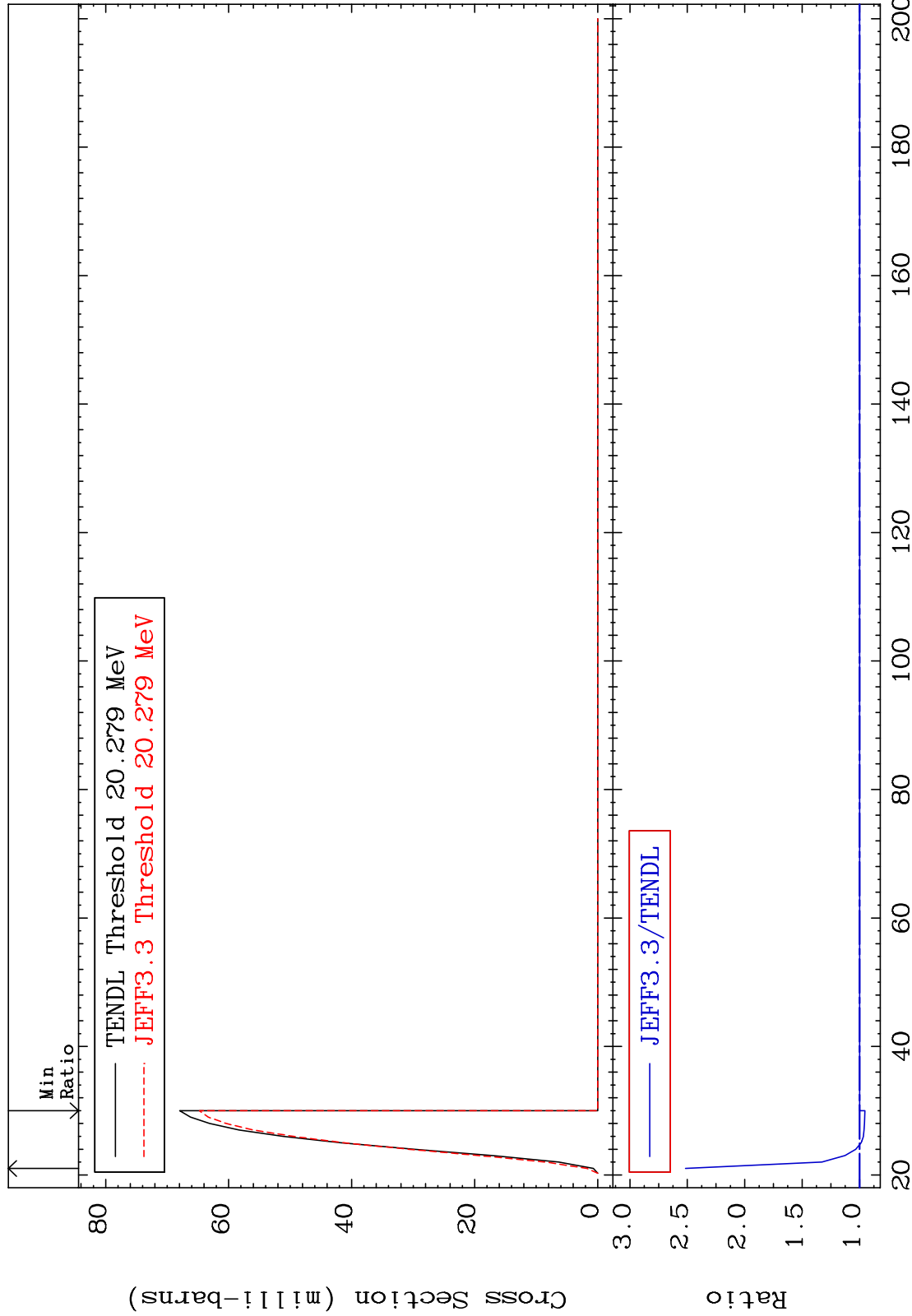
MAT 4122

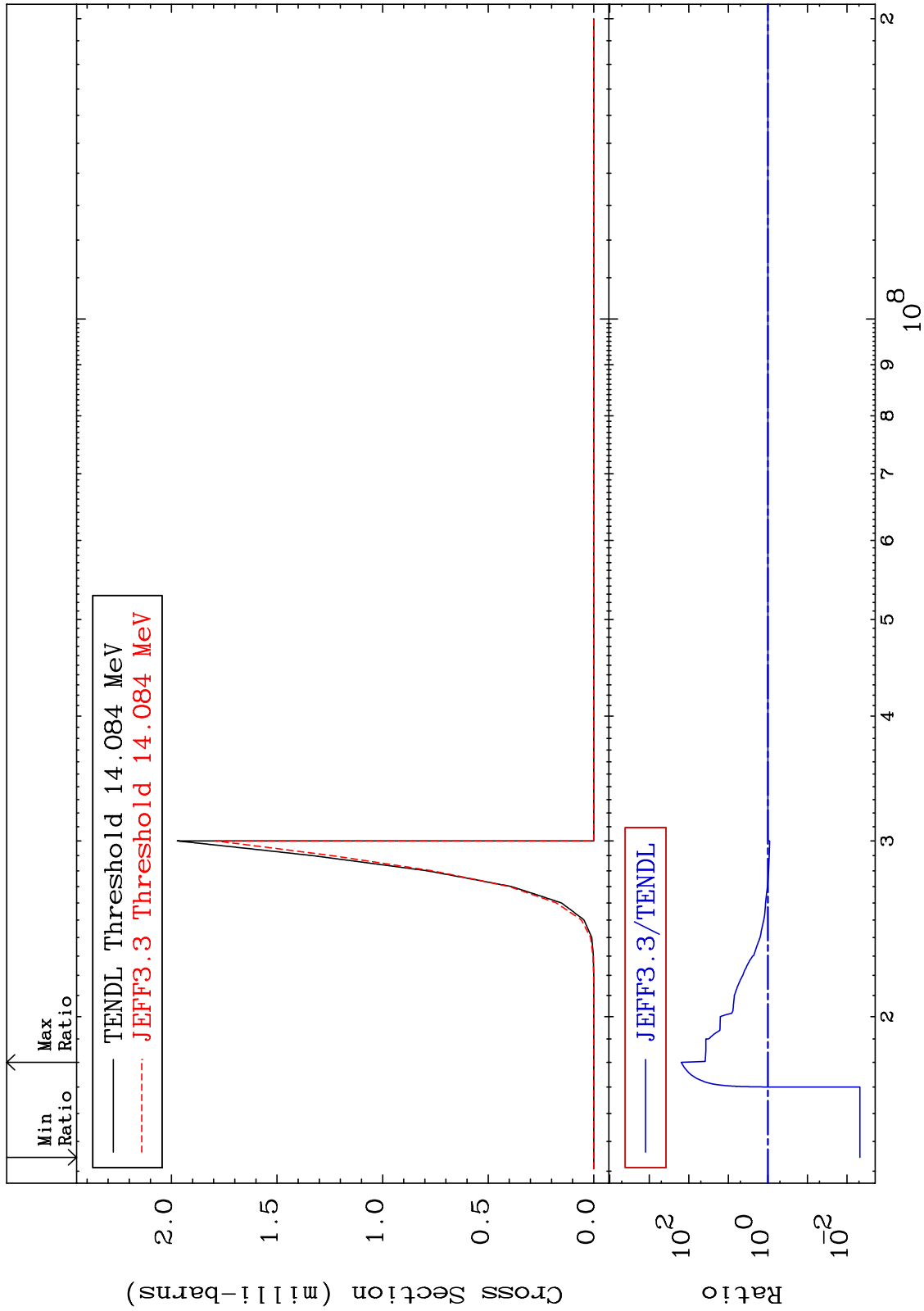
(n,3n):41-Nb-90m2

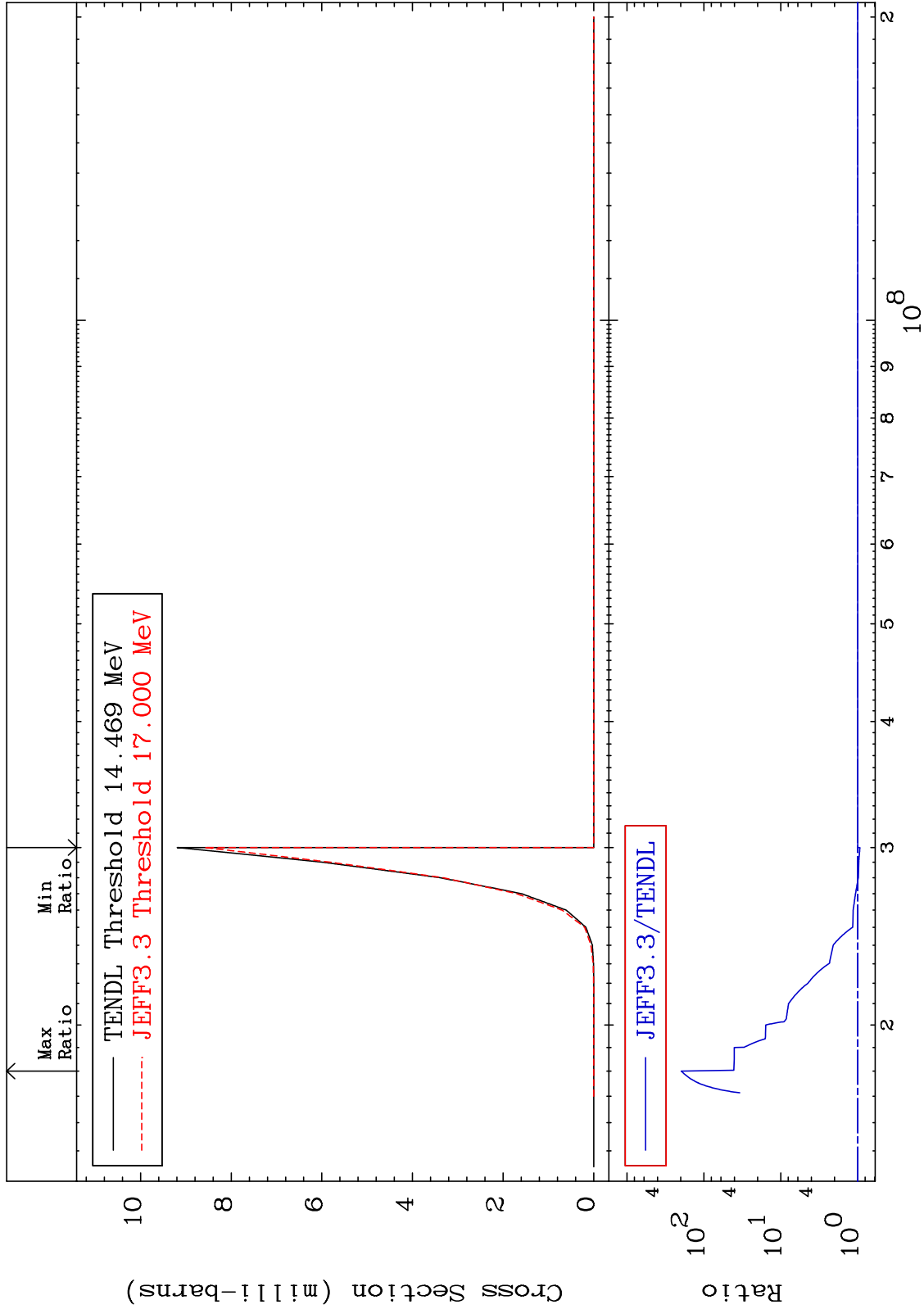
41-Nb-92

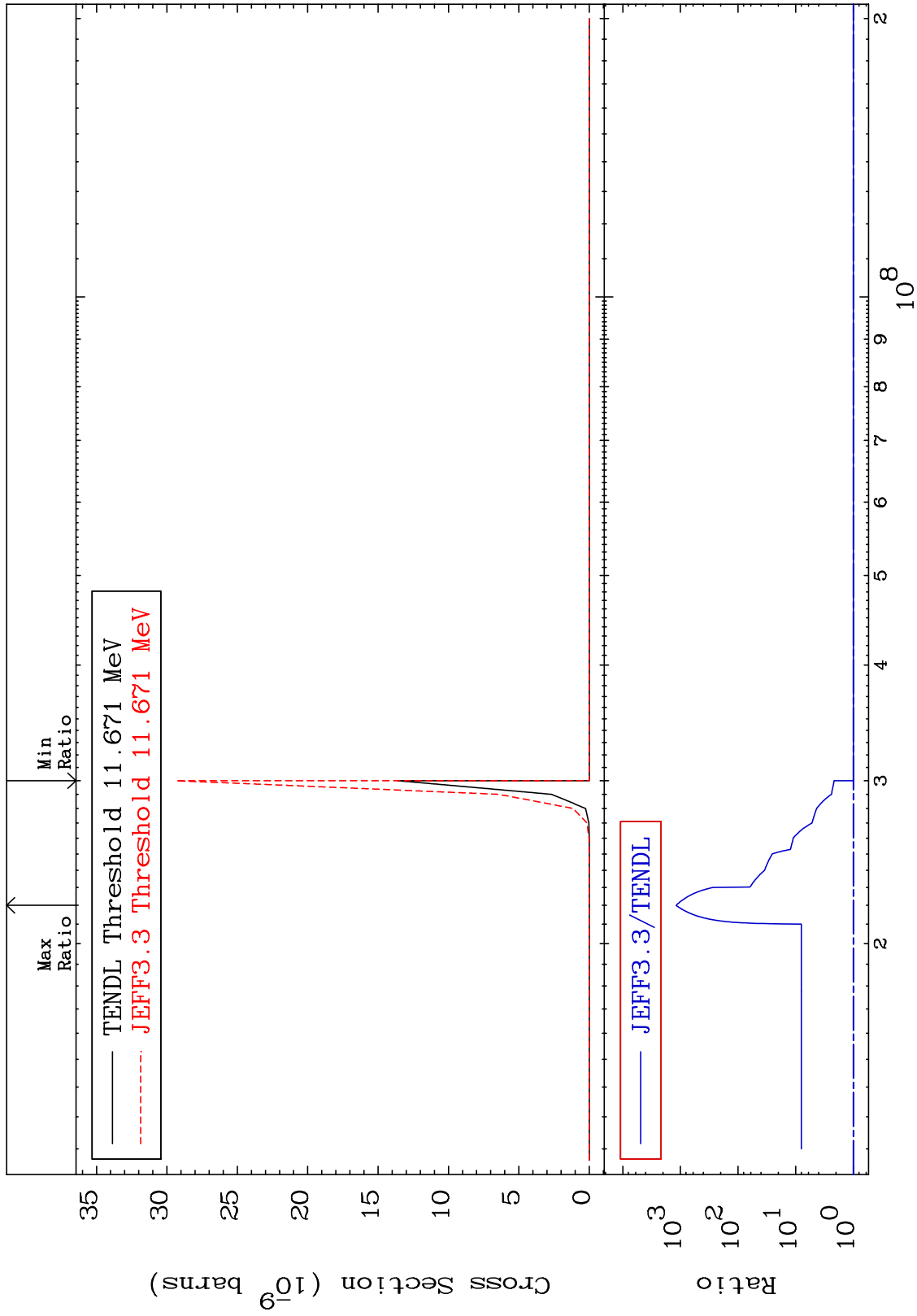
Radionuclide Production Cross Section

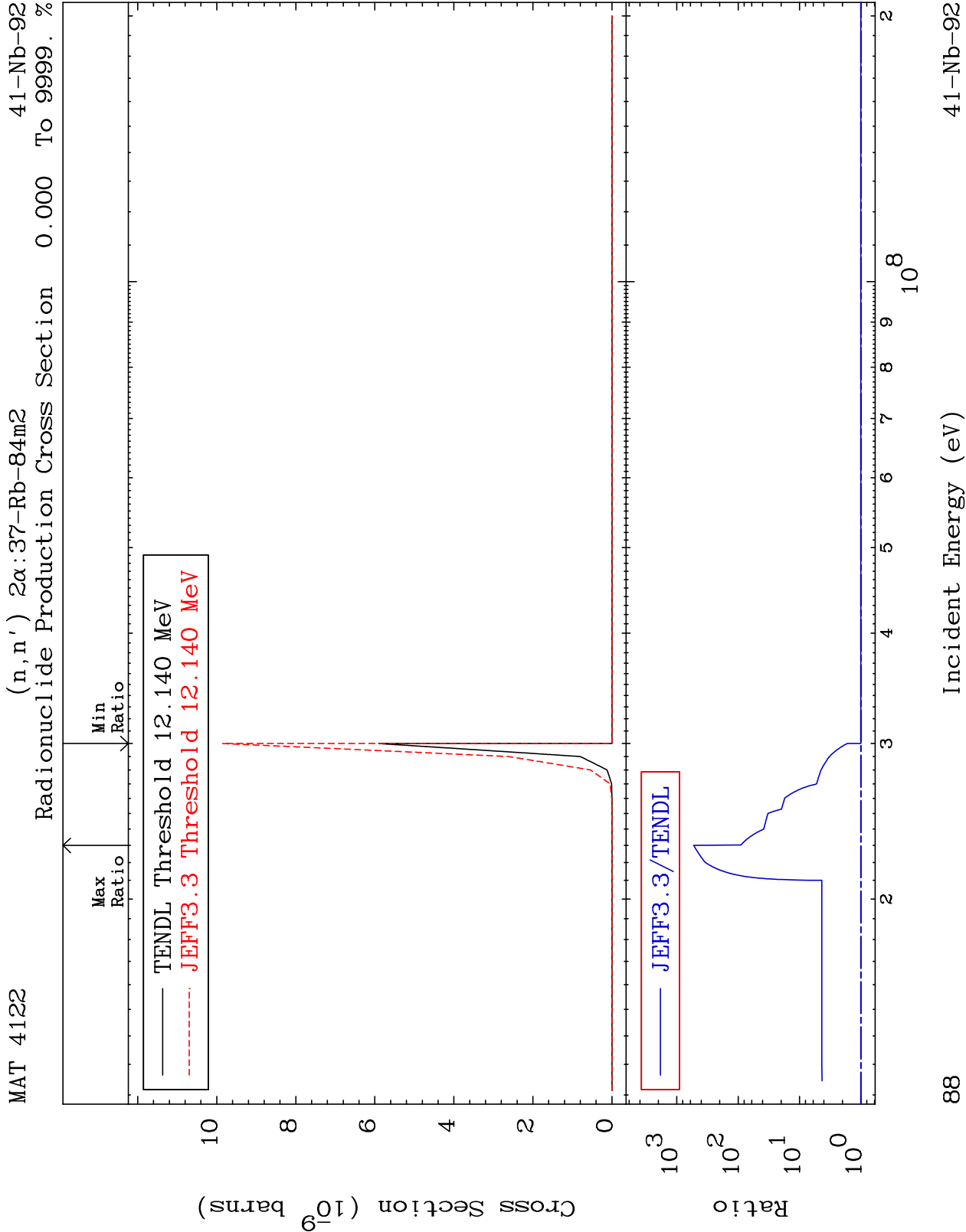
-4.711 To 151.6 %

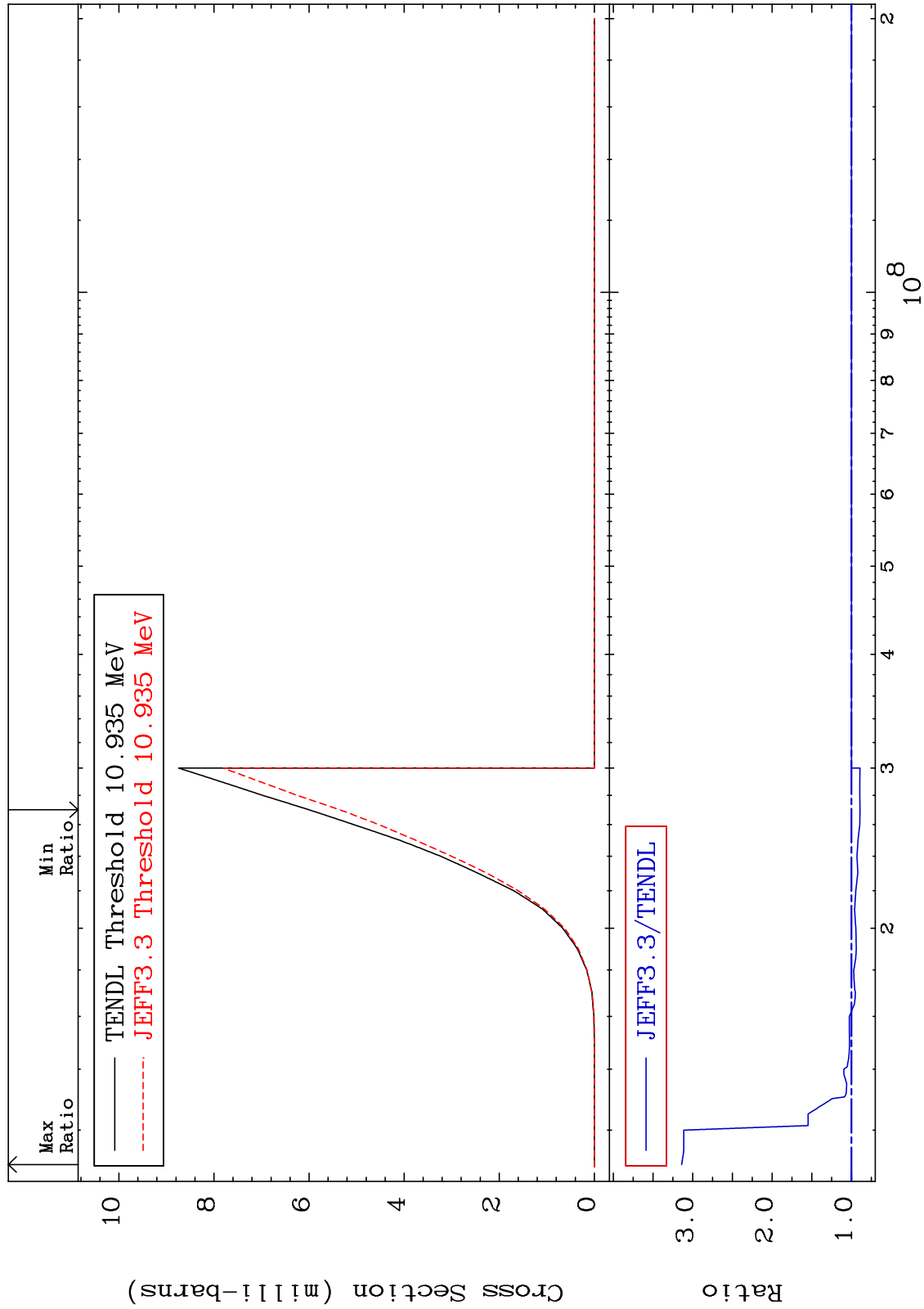












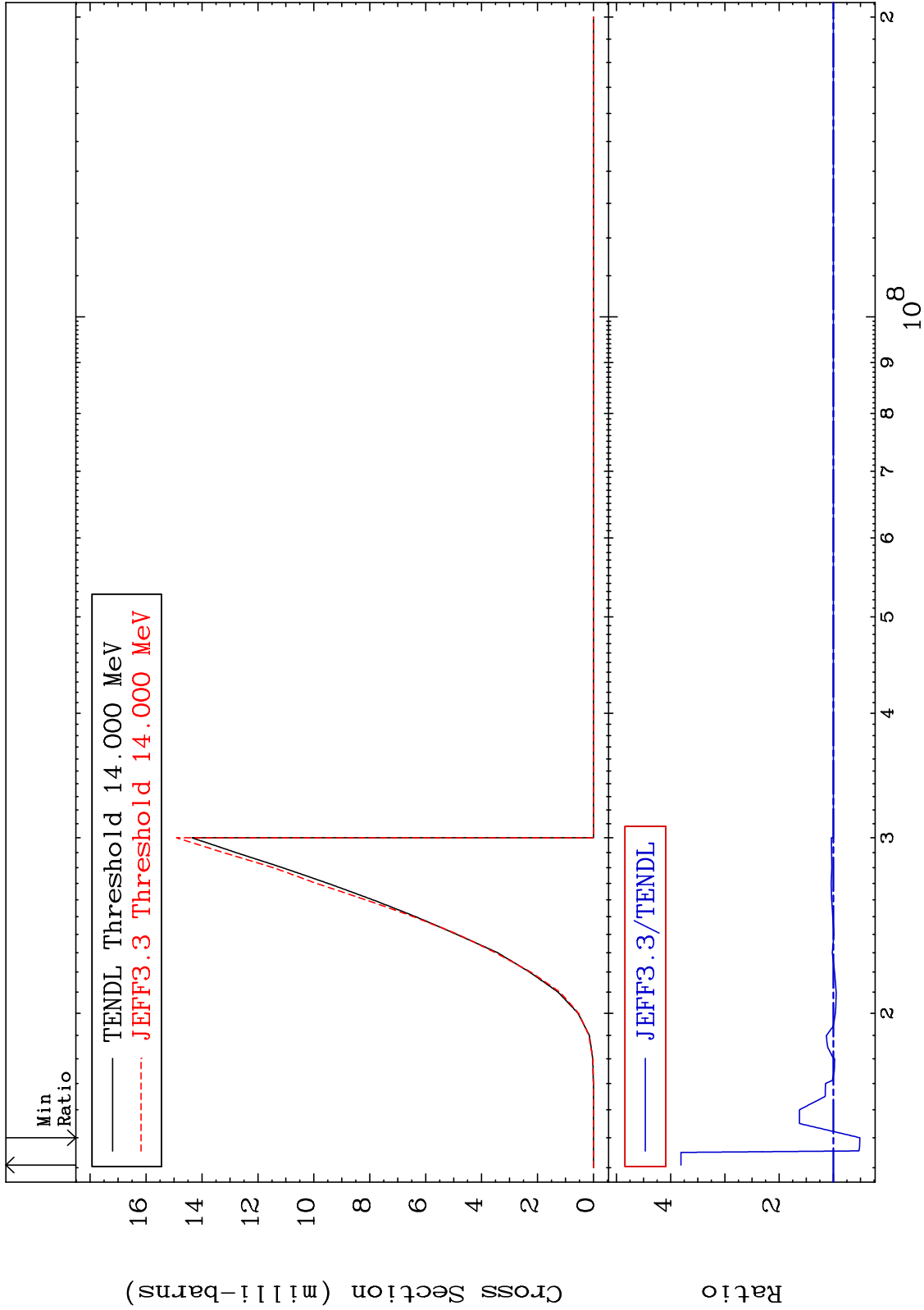
MAT 4122

(n,n') d:40-Zr-90m3

41-Nb-92

Radionuclide Production Cross Section

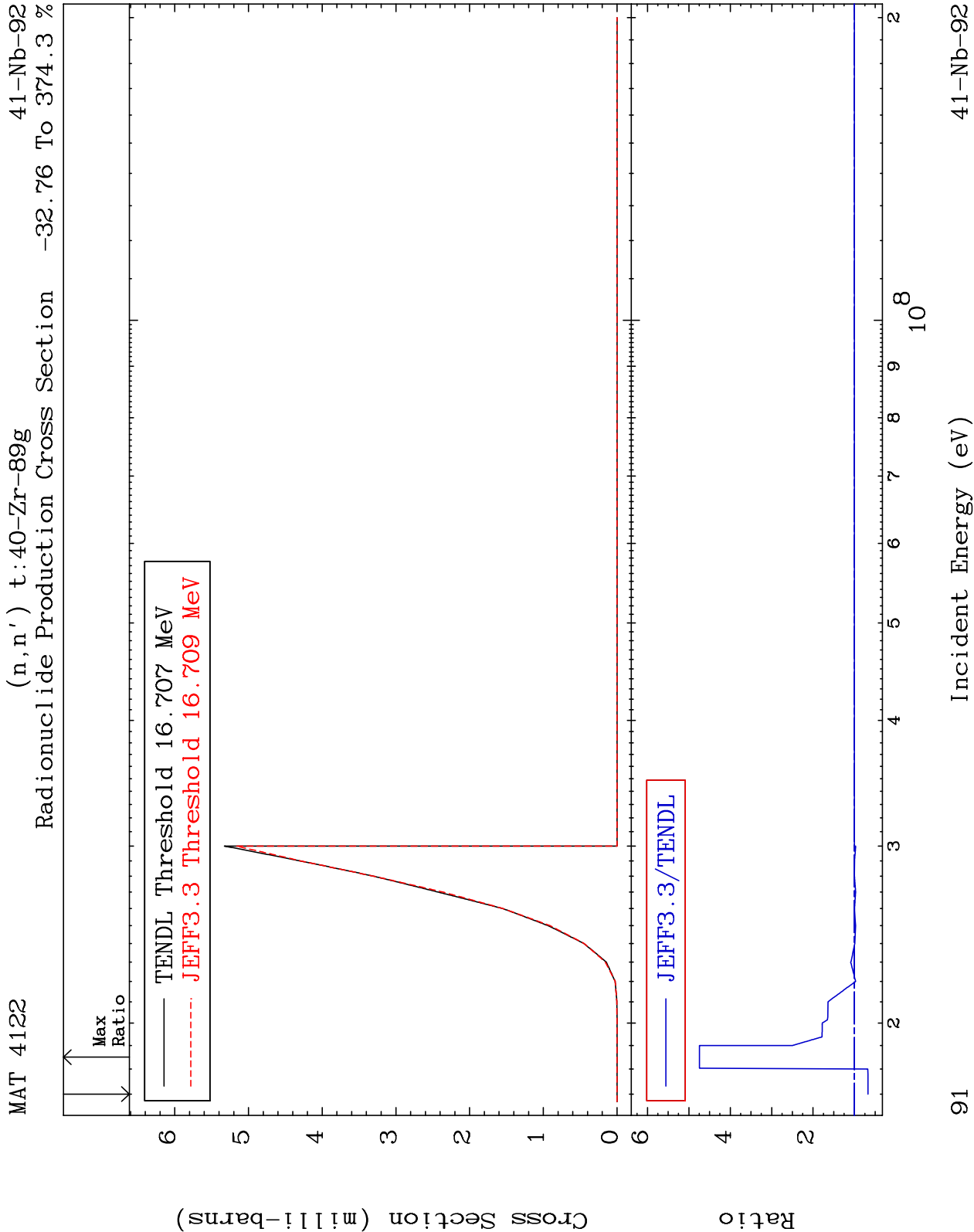
-48.71 To 281.0 %



90

Incident Energy (eV)

41-Nb-92

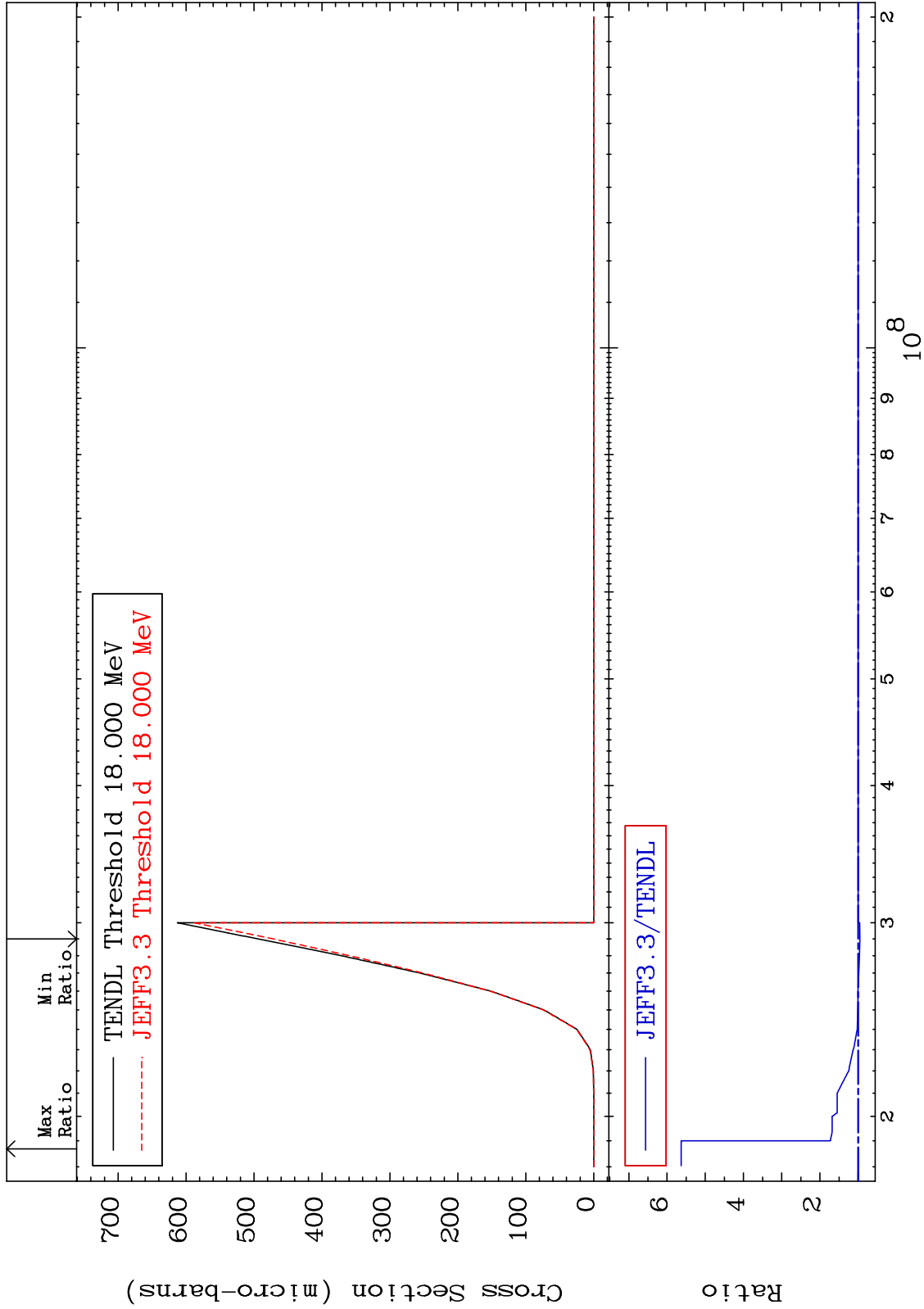


MAT 4122

(n,n') t:40-Zr-89m1

41-Nb-92

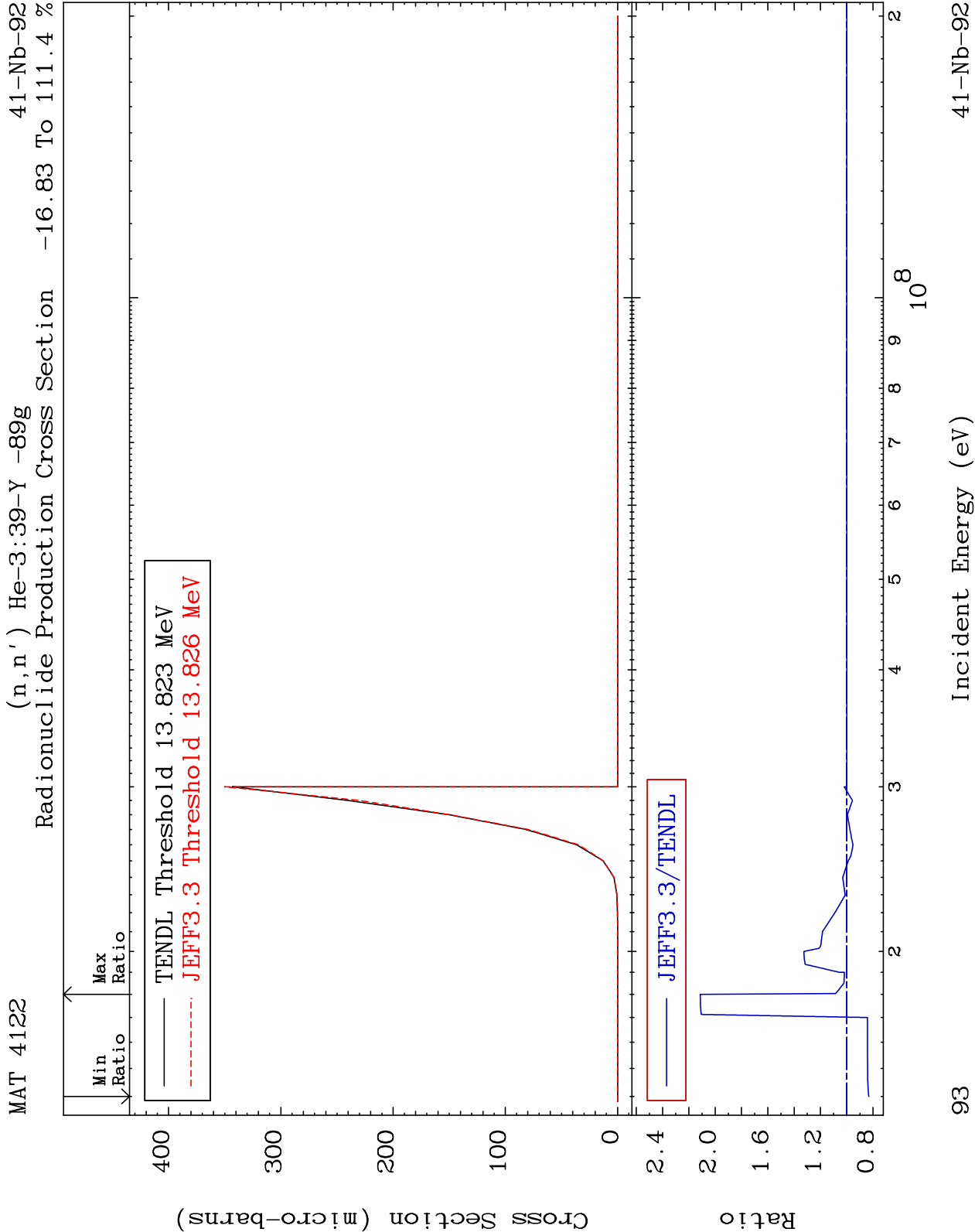
Radionuclide Production Cross Section -4.541 To 463.0 %



92

Incident Energy (eV)

41-Nb-92

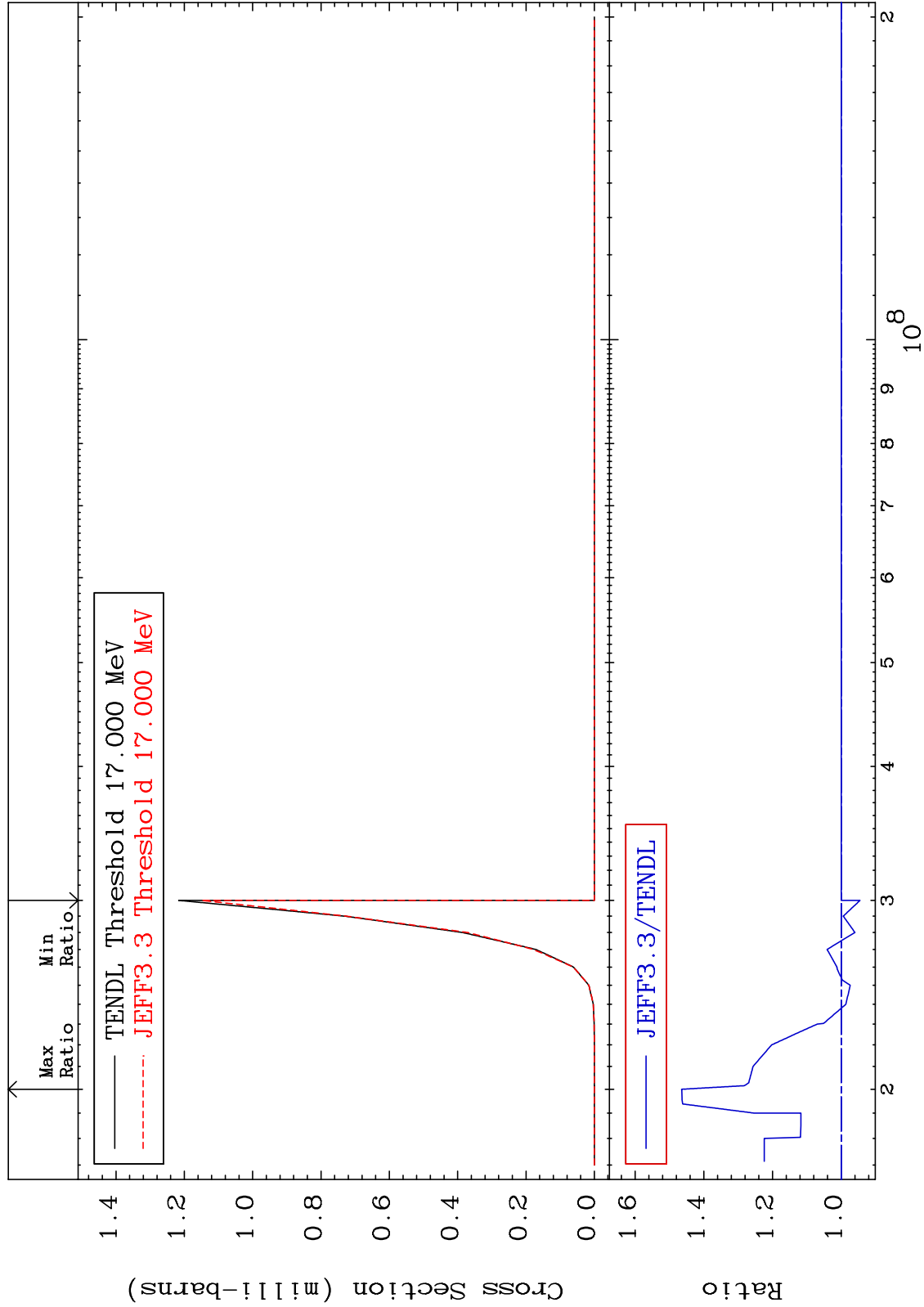


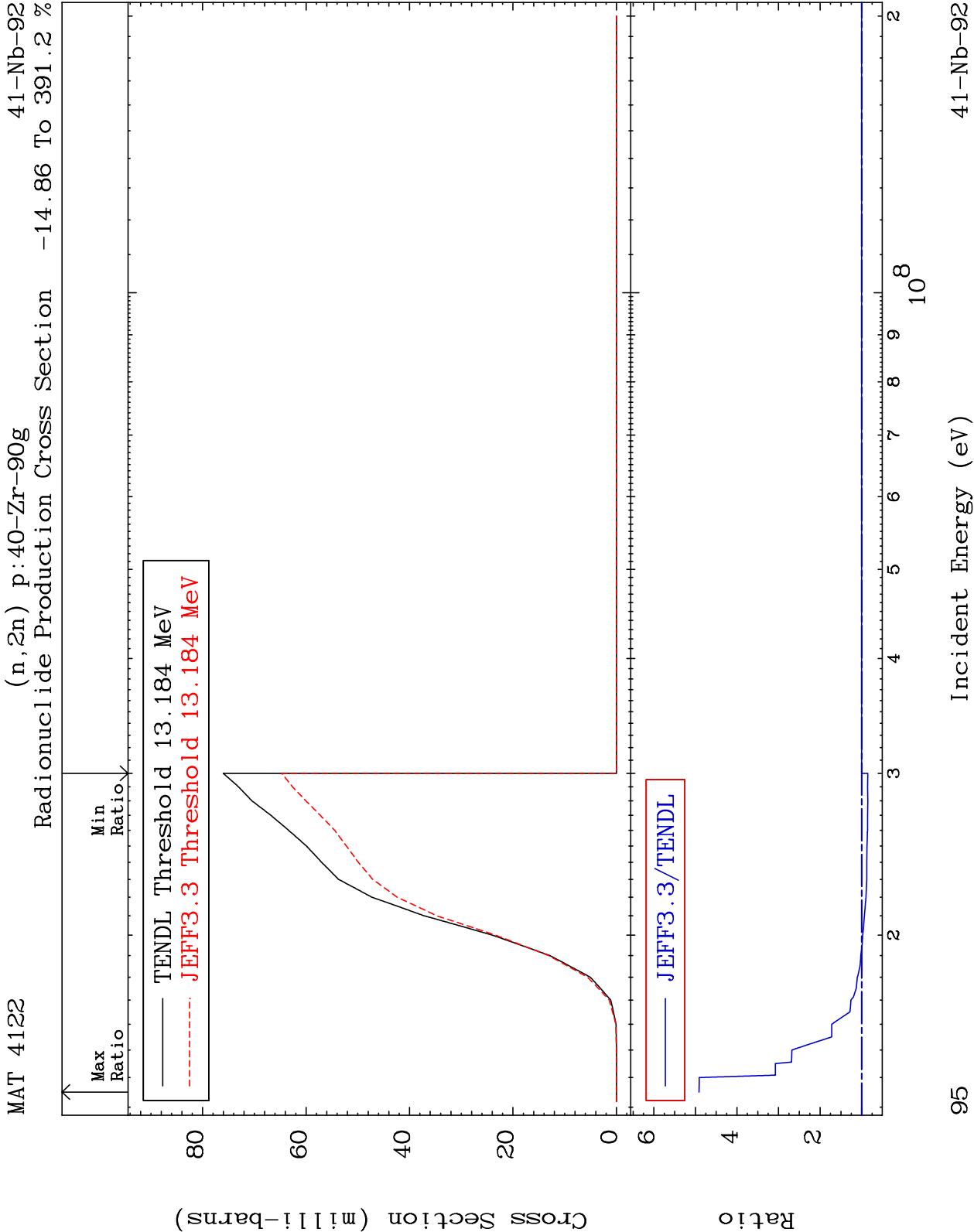
MAT 4122

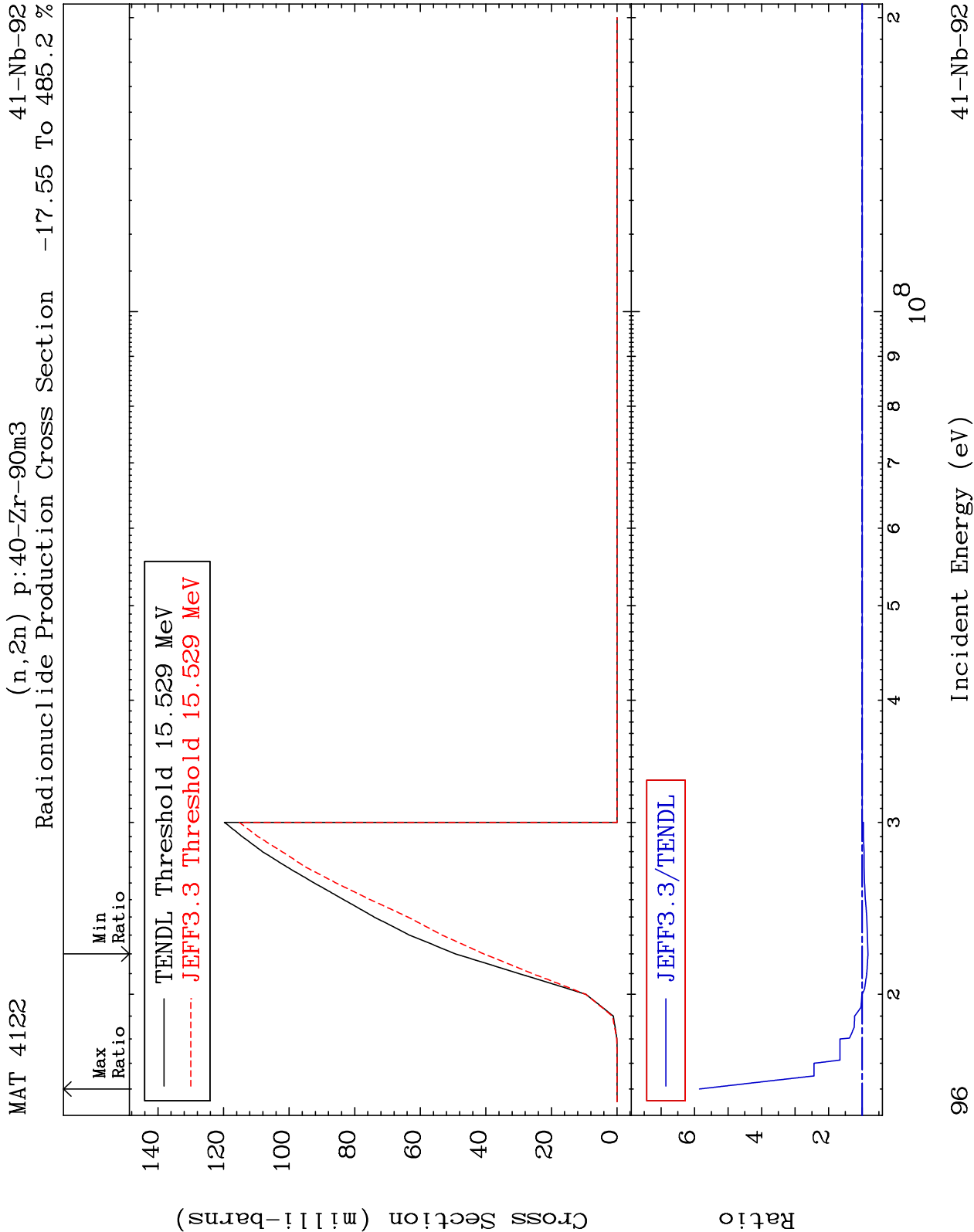
(n,n') He-3:39-Y -89m1

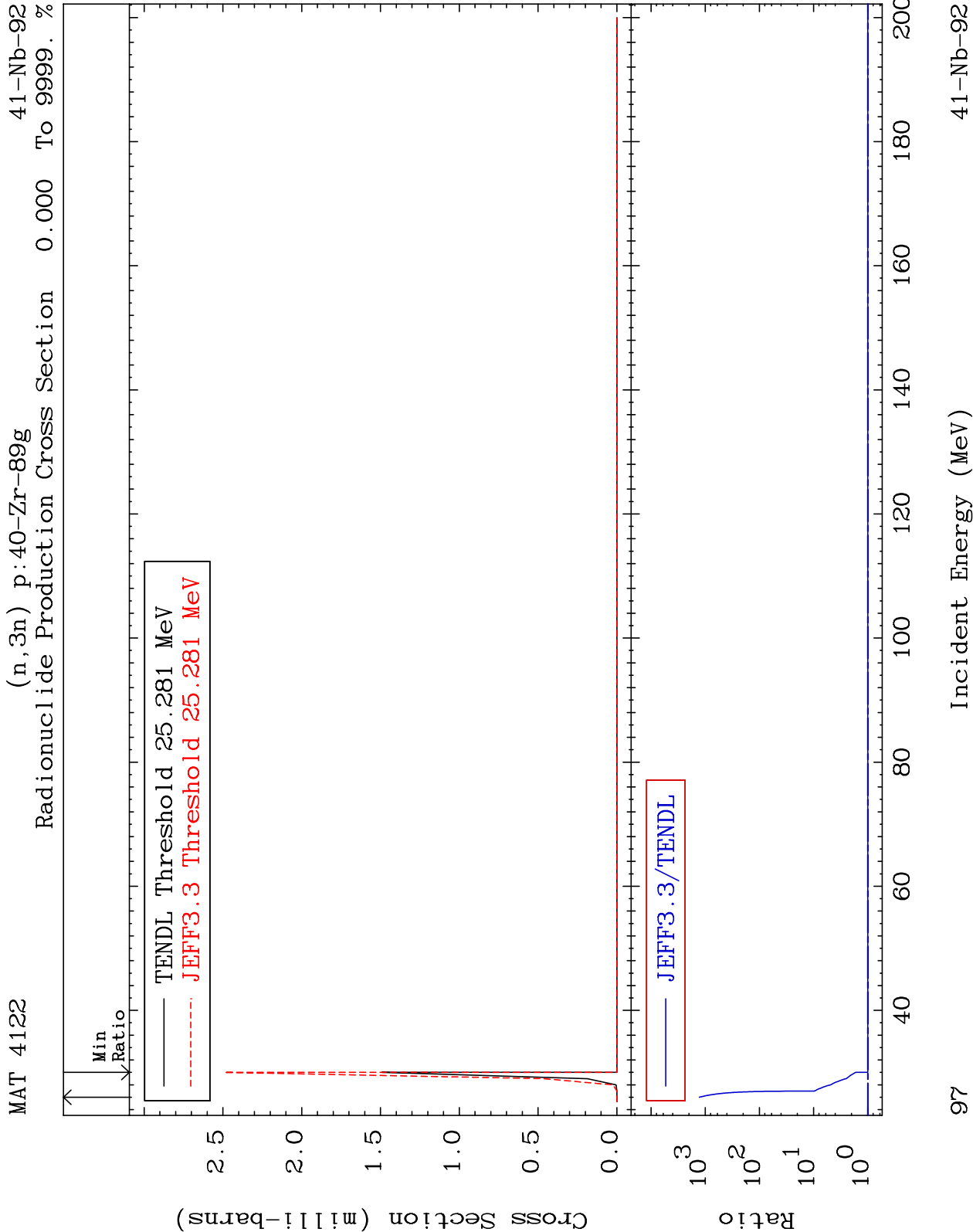
41-Nb-92

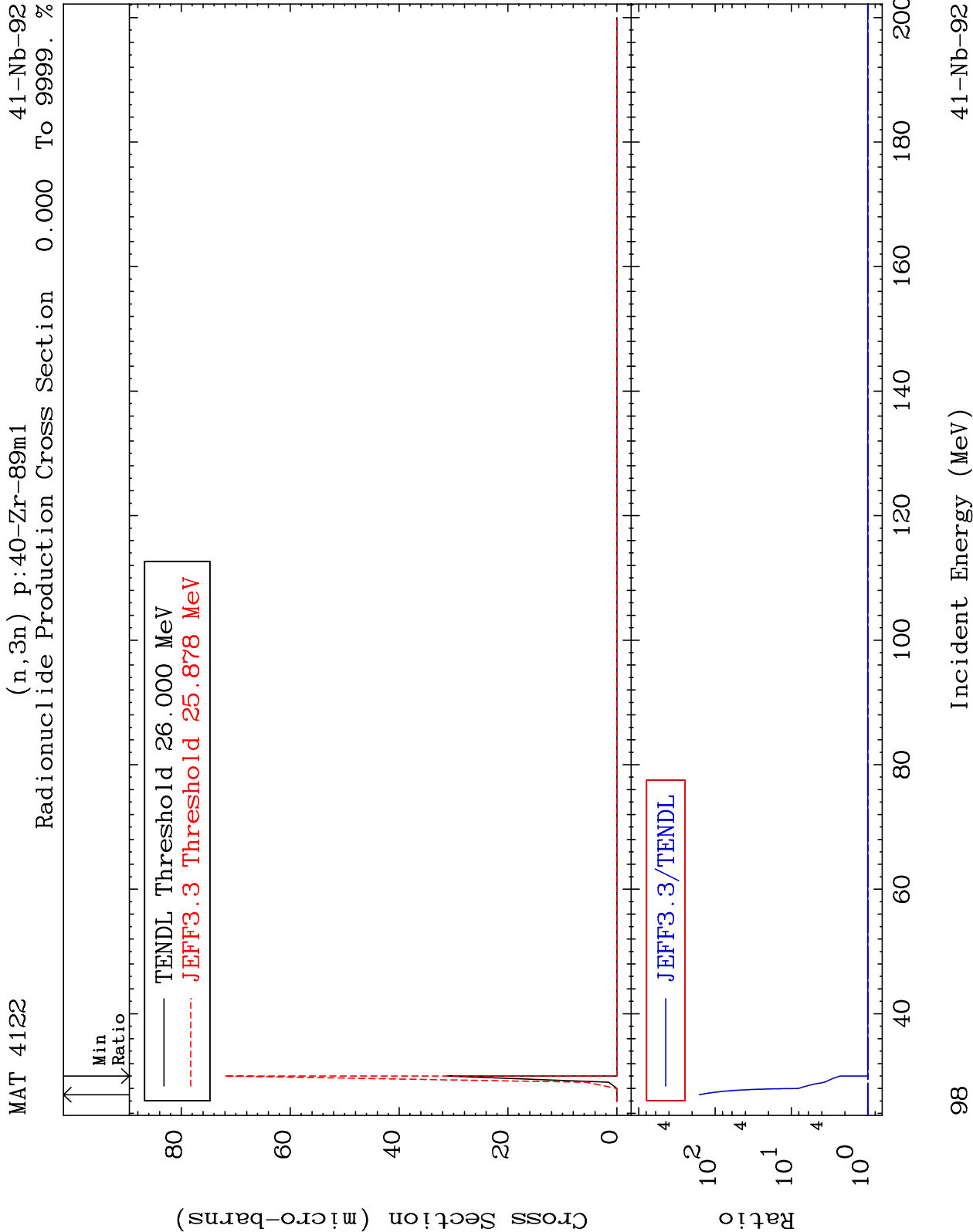
Radionuclide Production Cross Section -5.455 To 46.47 %

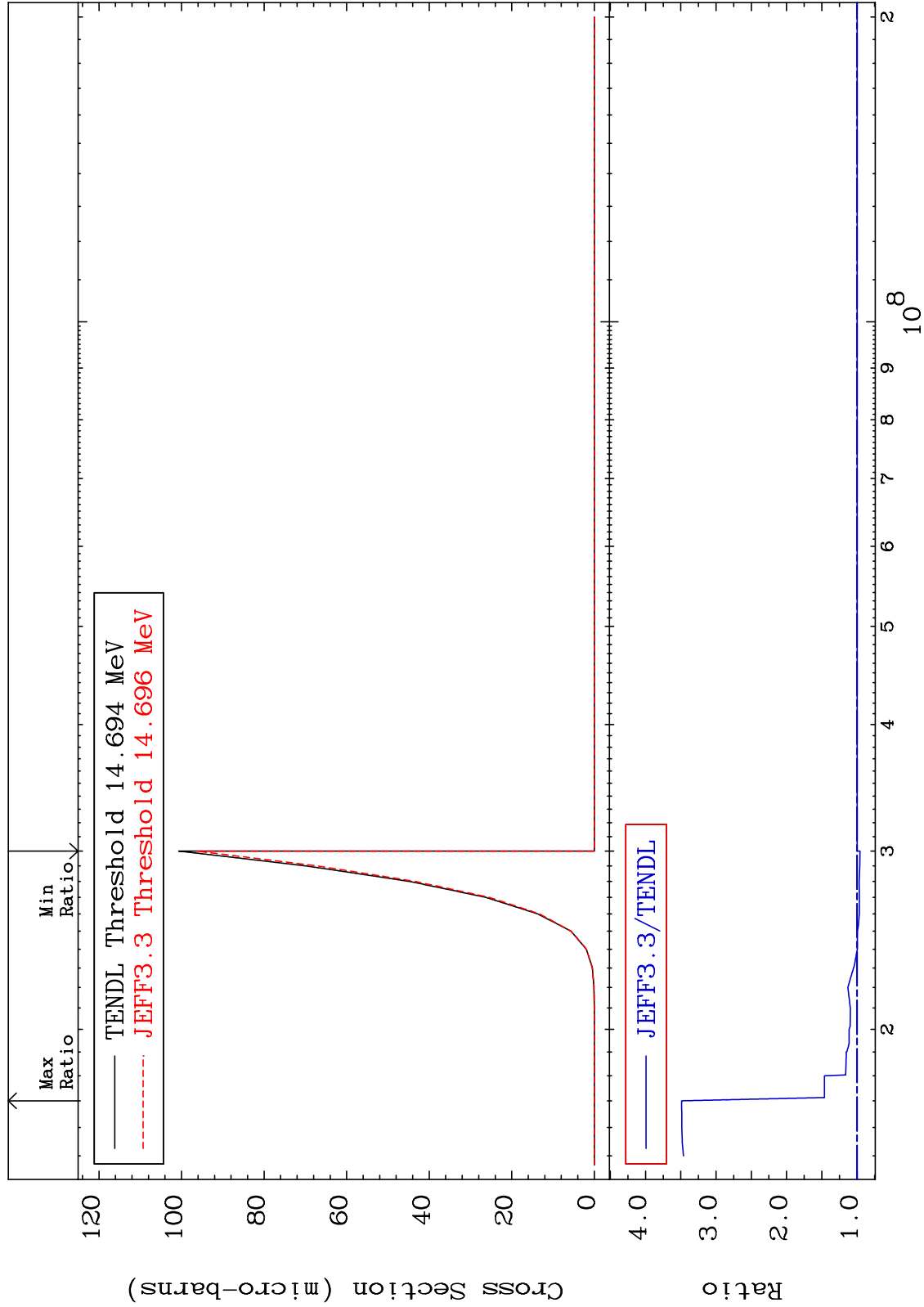


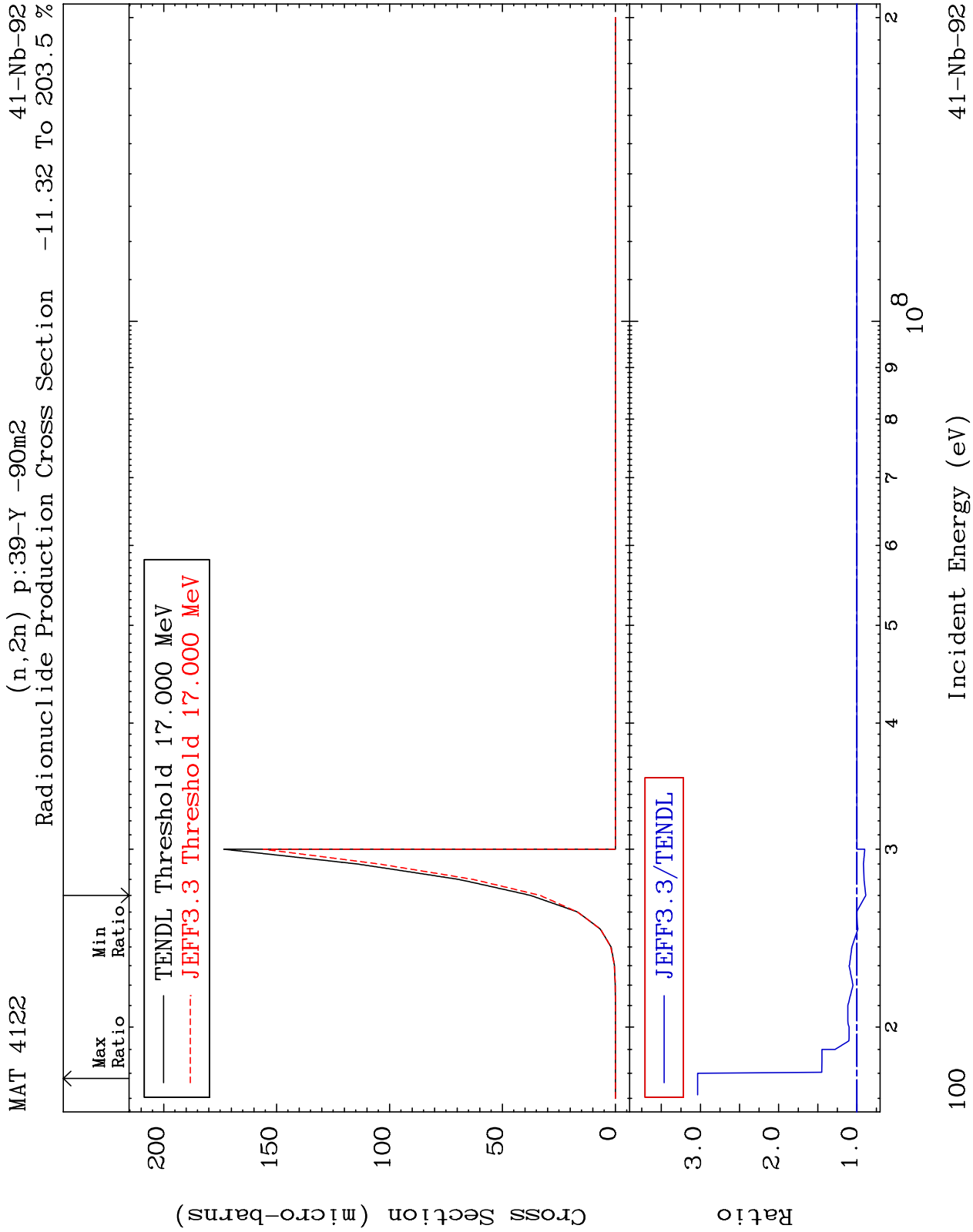


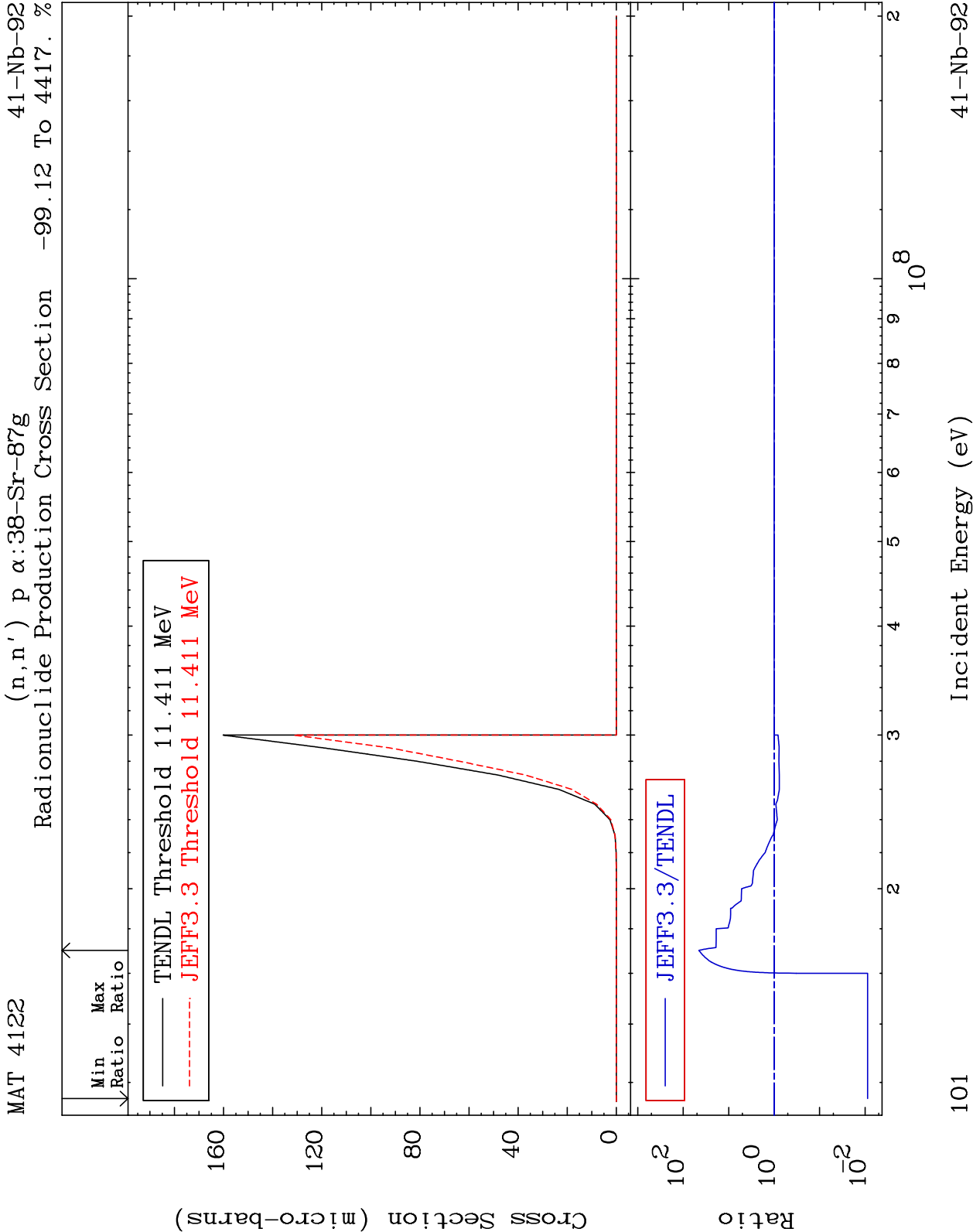


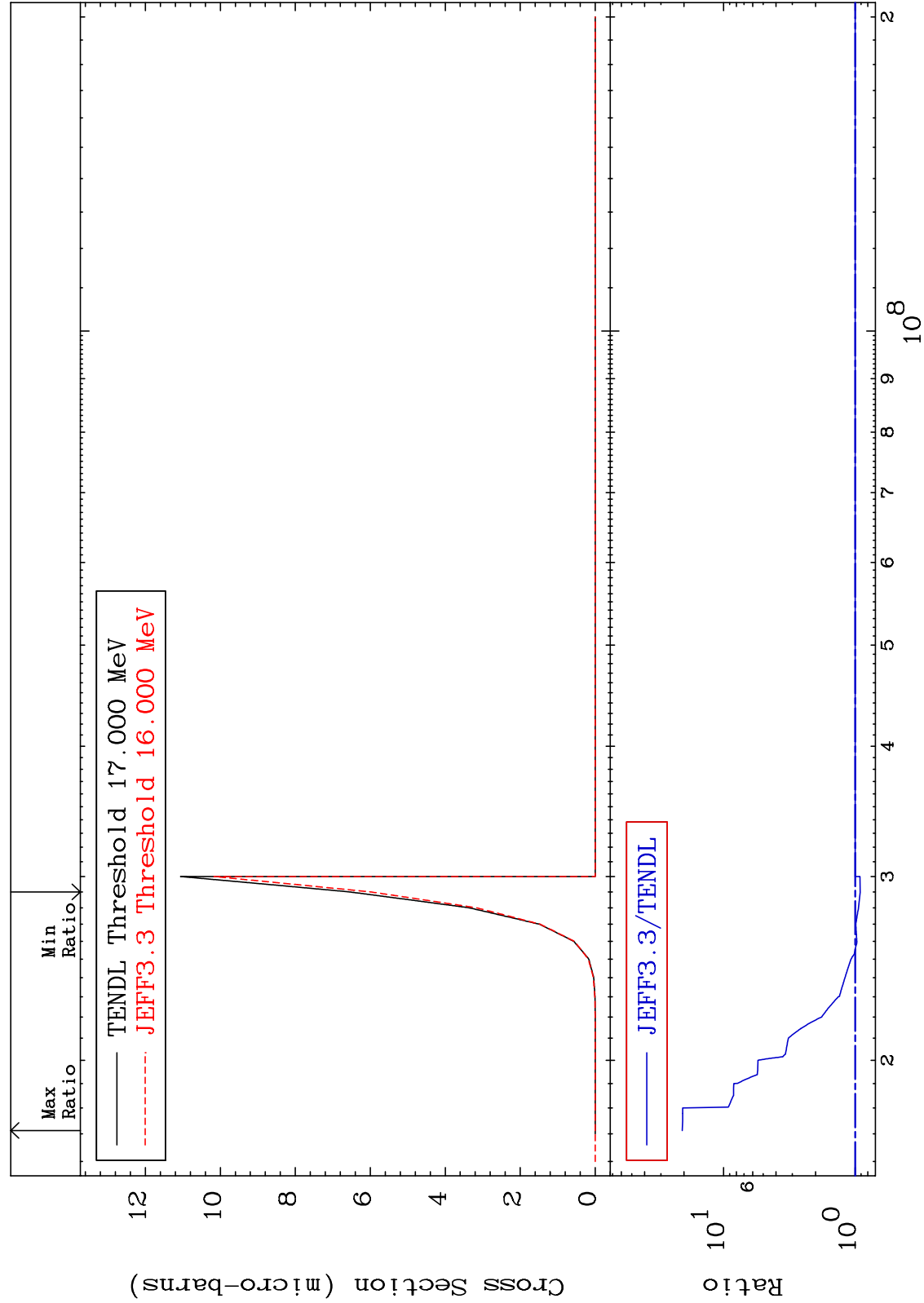










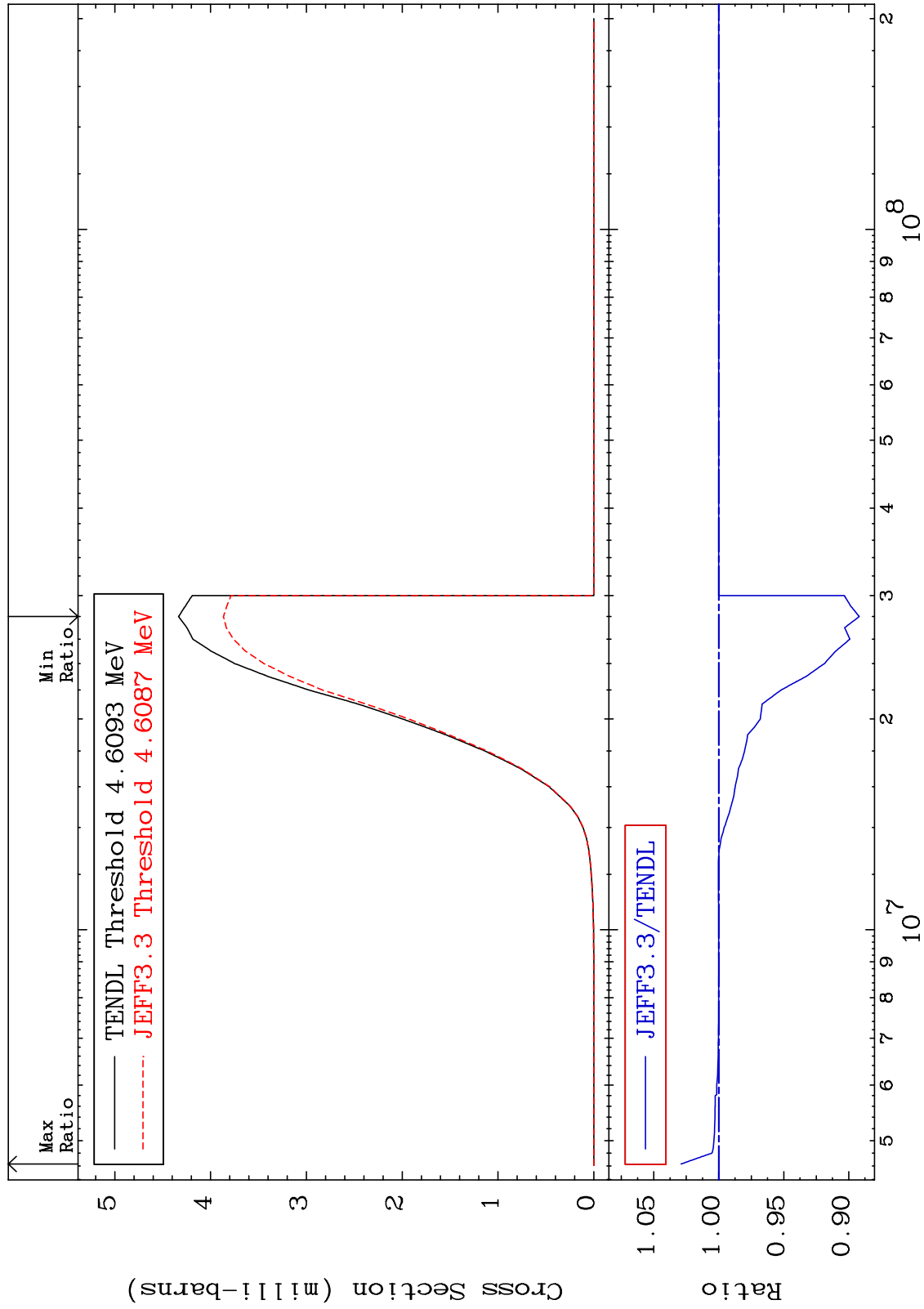


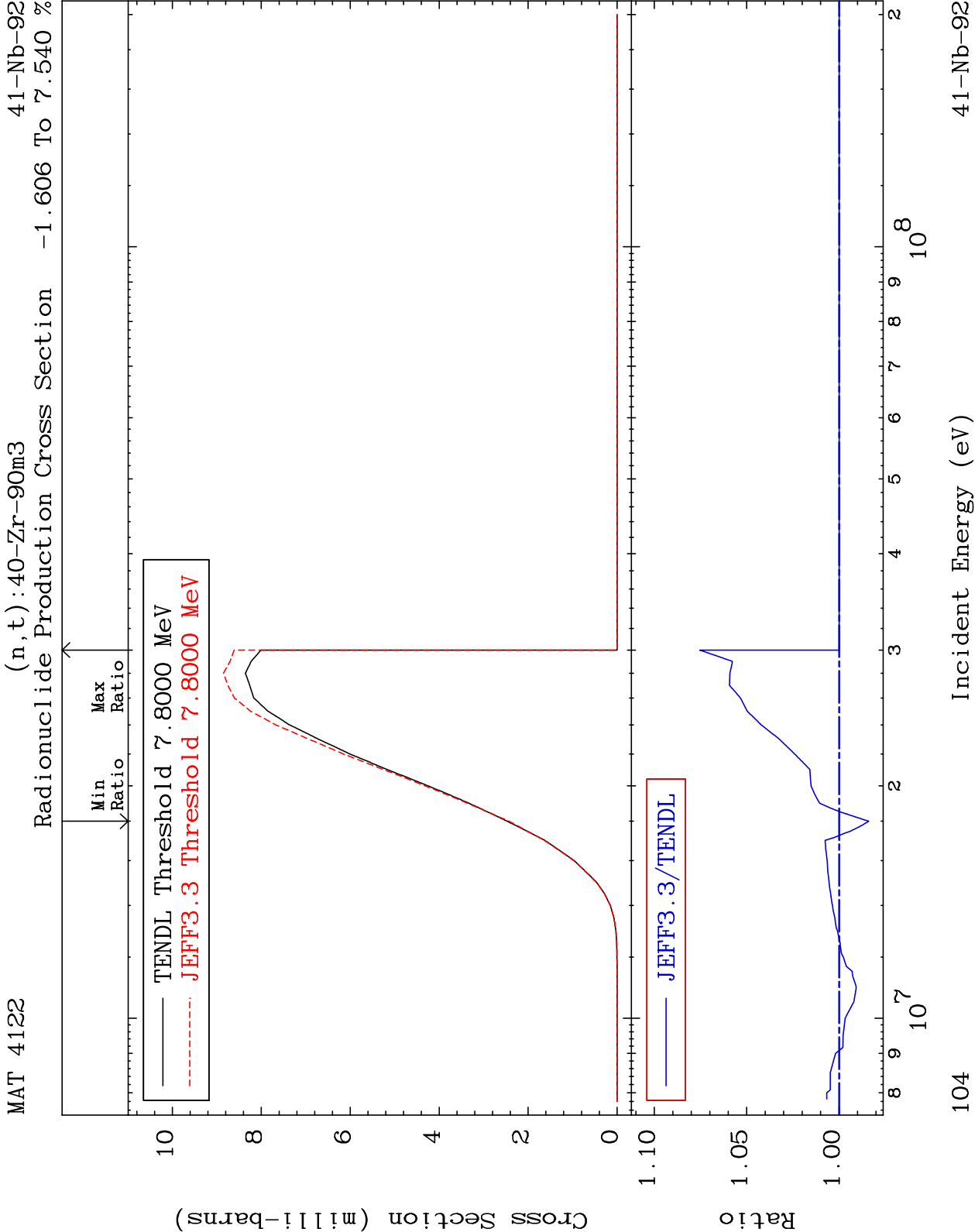
MAT 4122

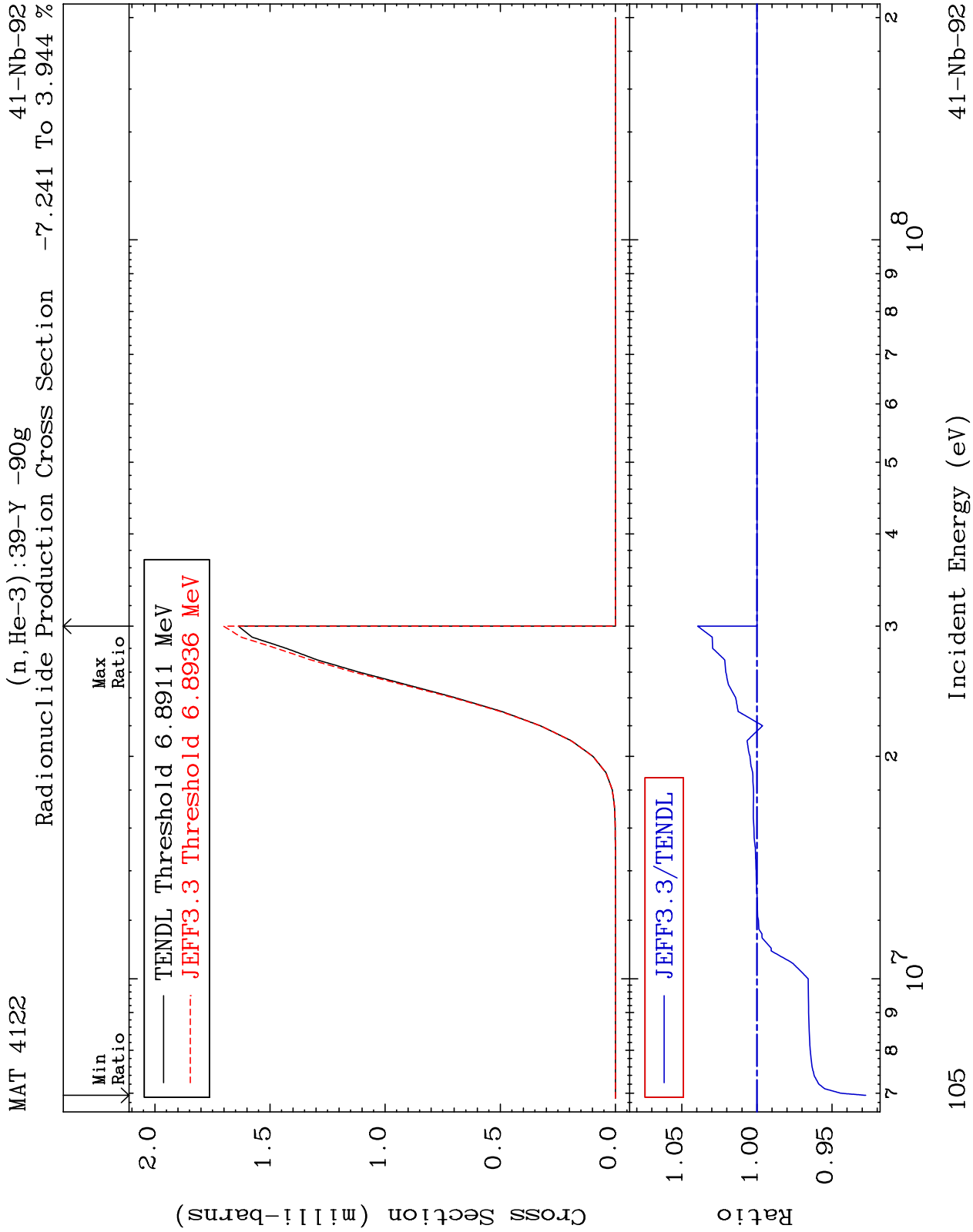
(n,t):40-Zr-90g

41-Nb-92

Radionuclide Production Cross Section -10.81 To 2.904 %







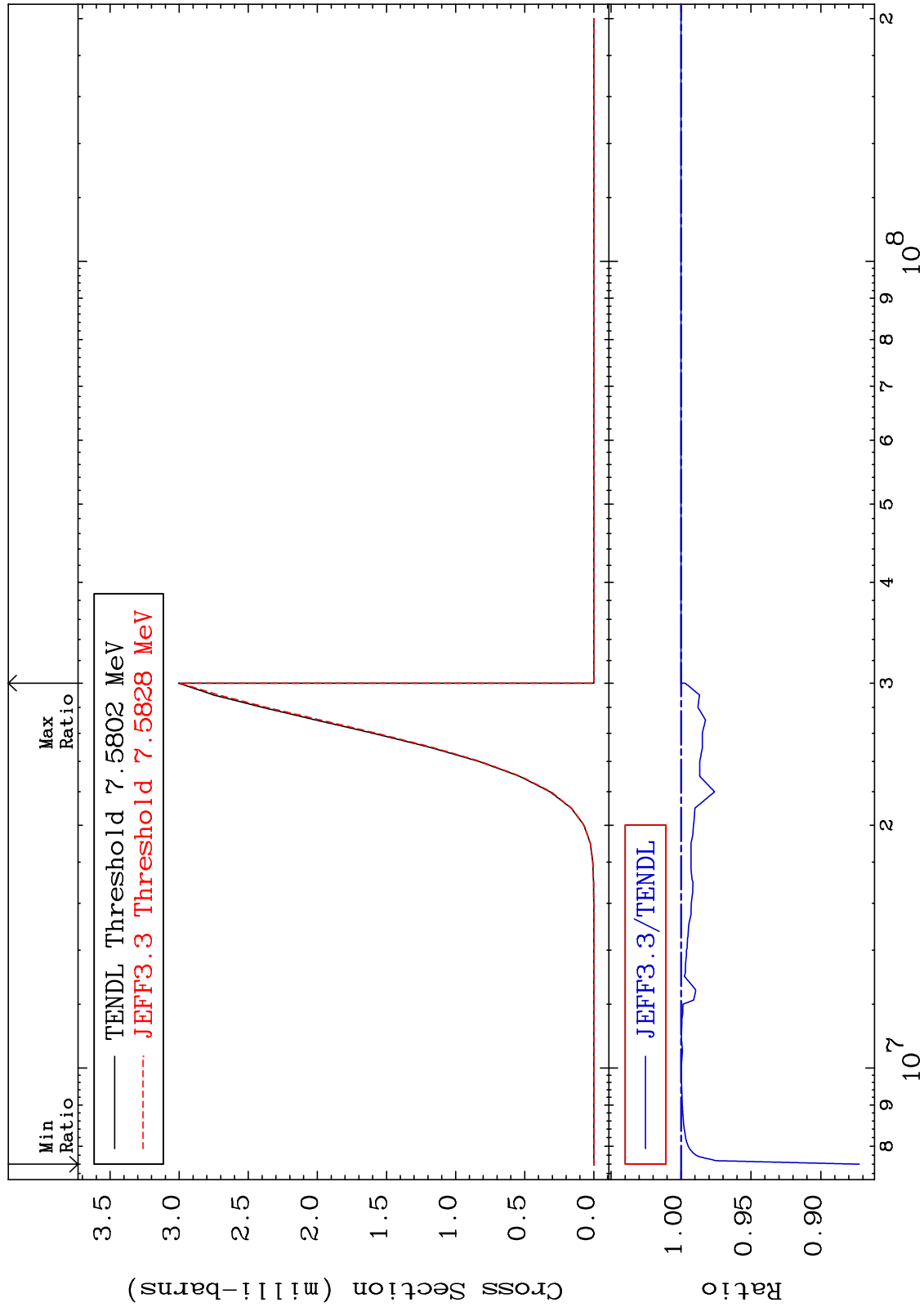
MAT 4122

(n,He-3):39-Y -90m2

41-Nb-92

Radionuclide Production Cross Section

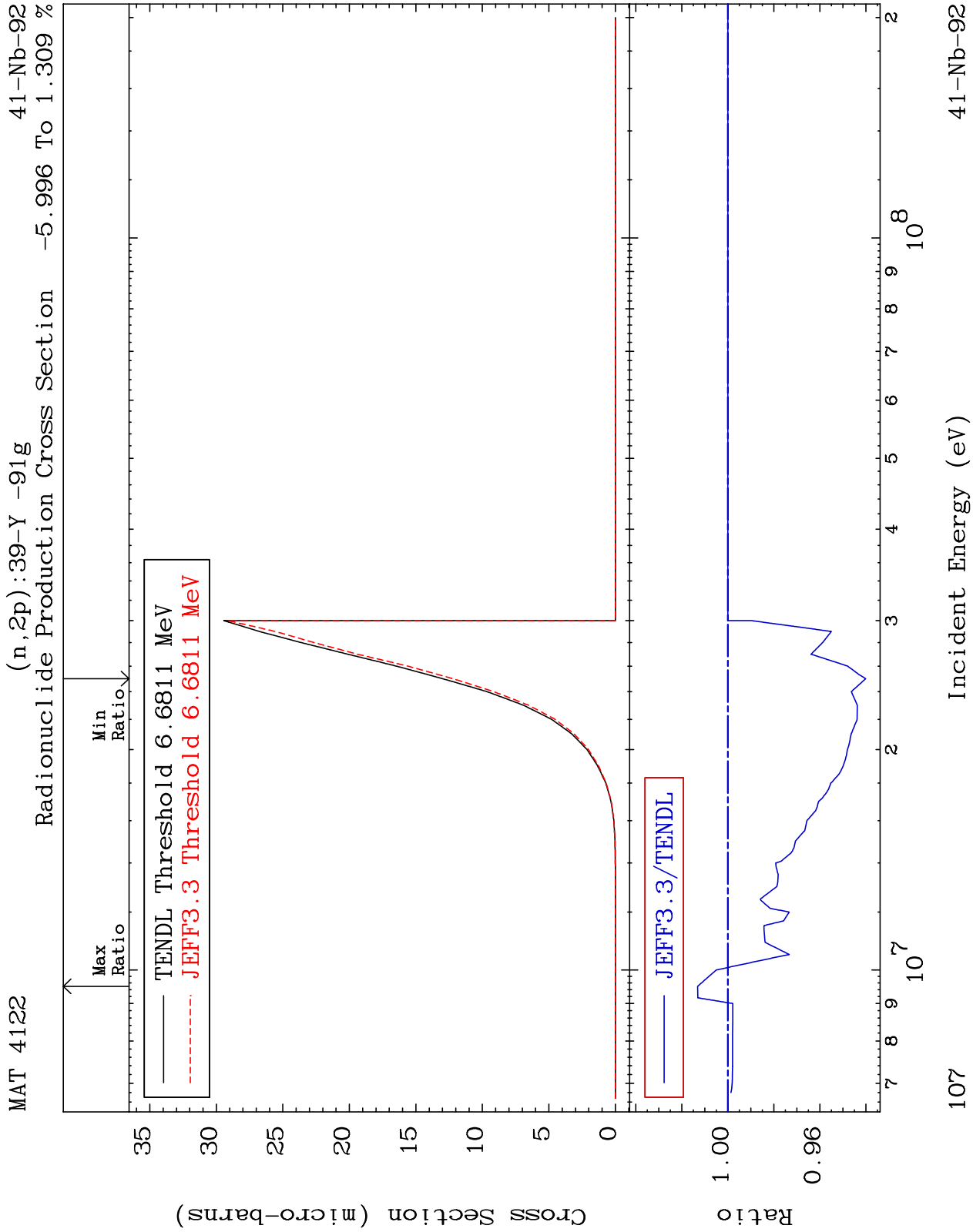
-12.75 To 0.000 %

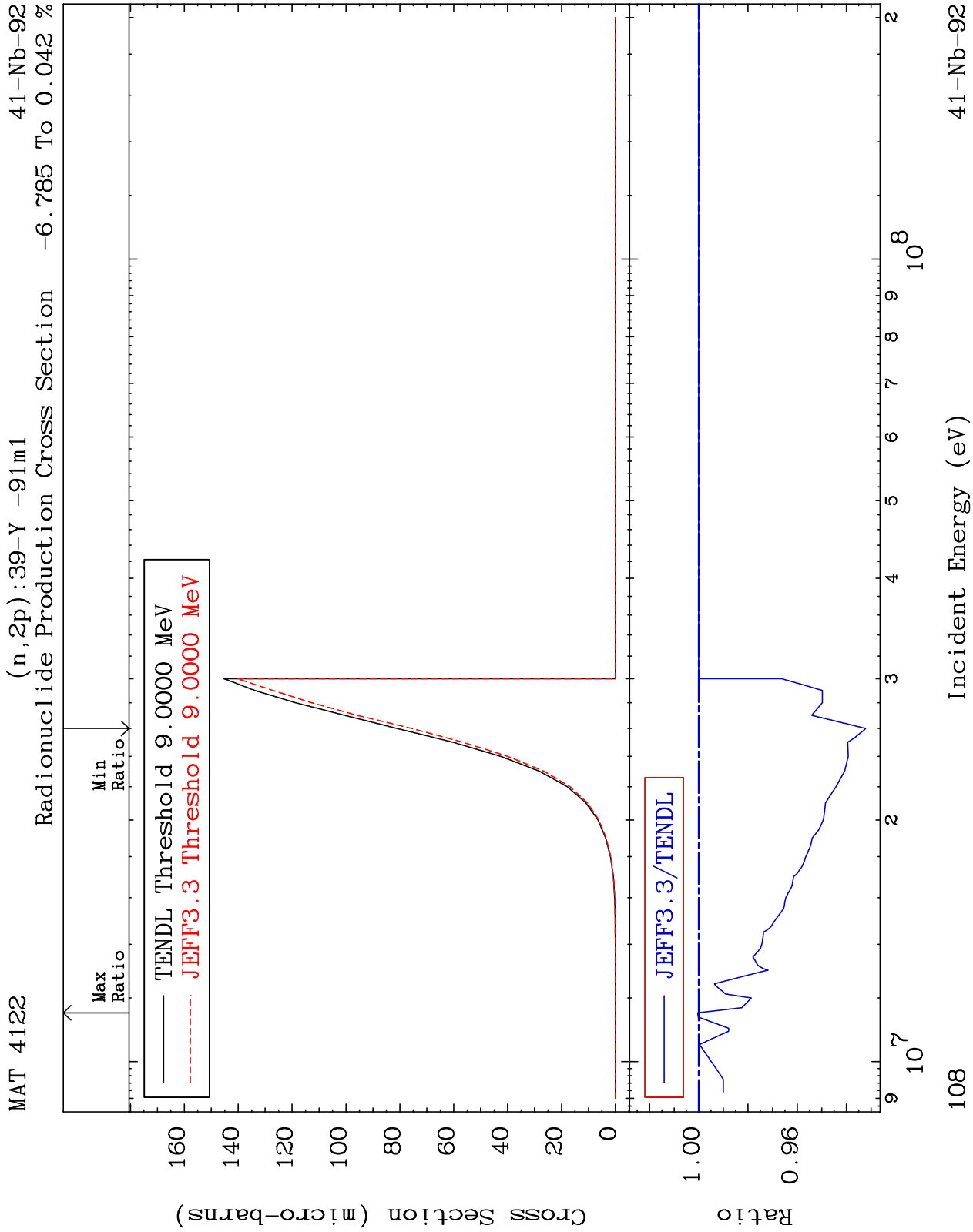


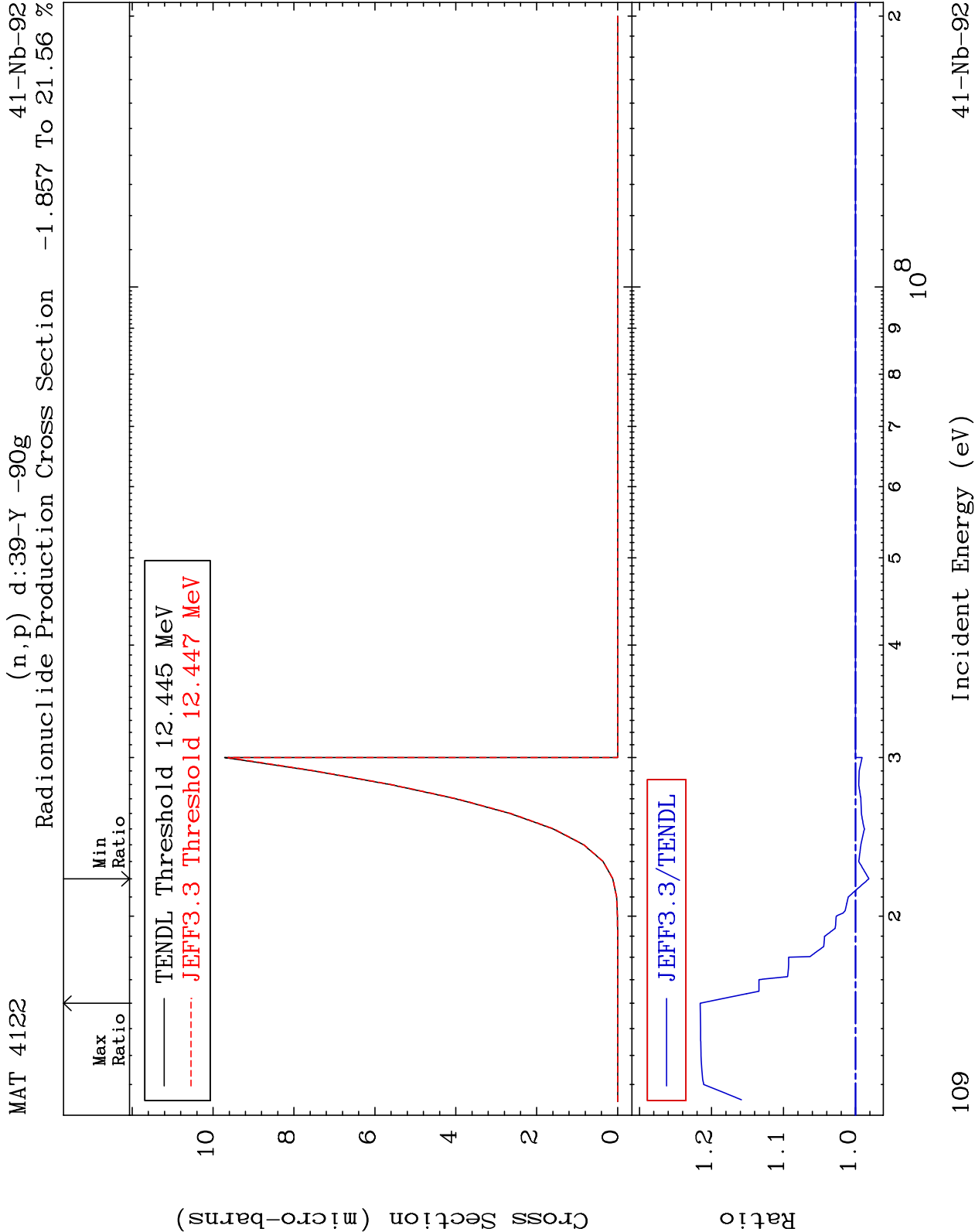
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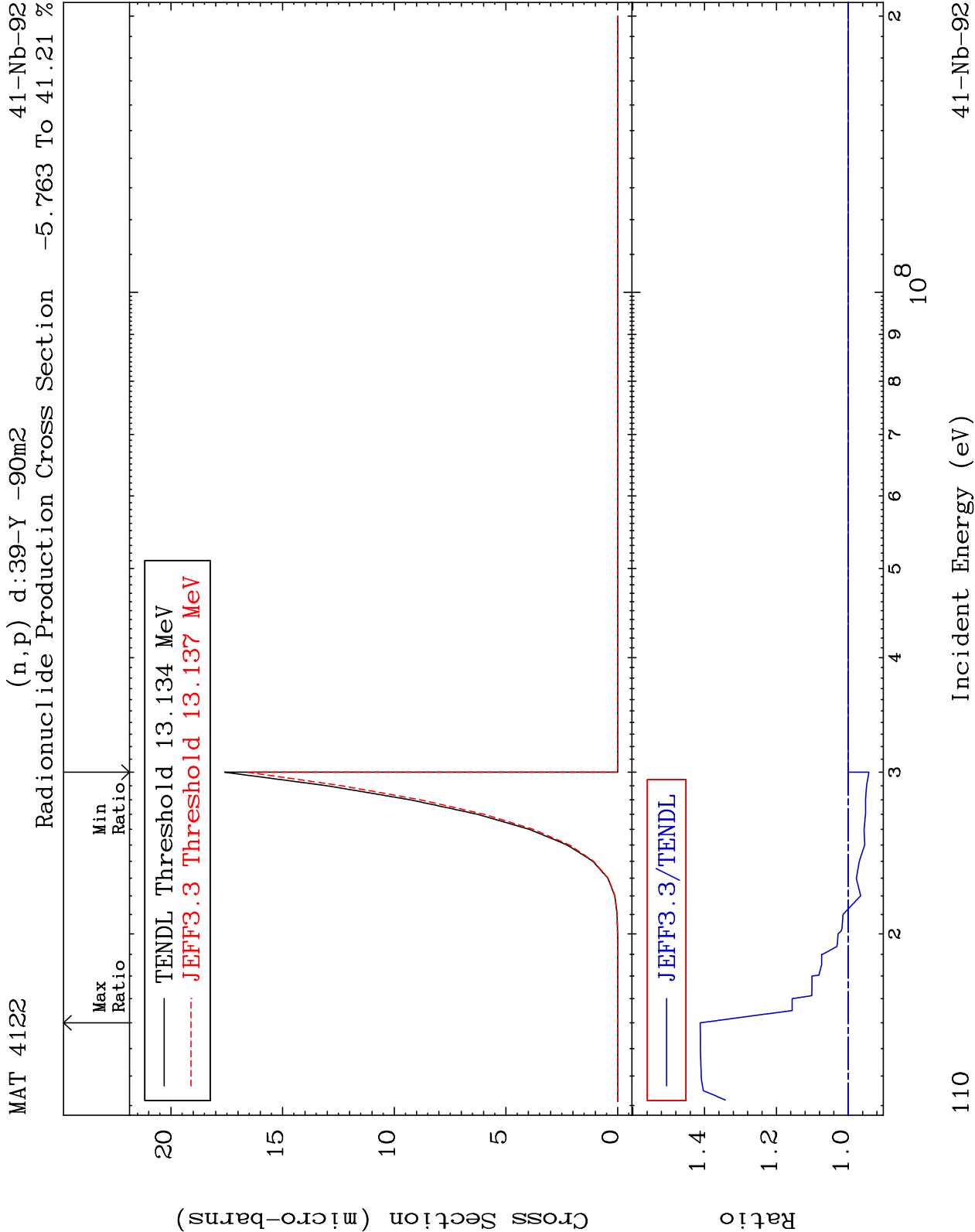
Incident Energy (eV)

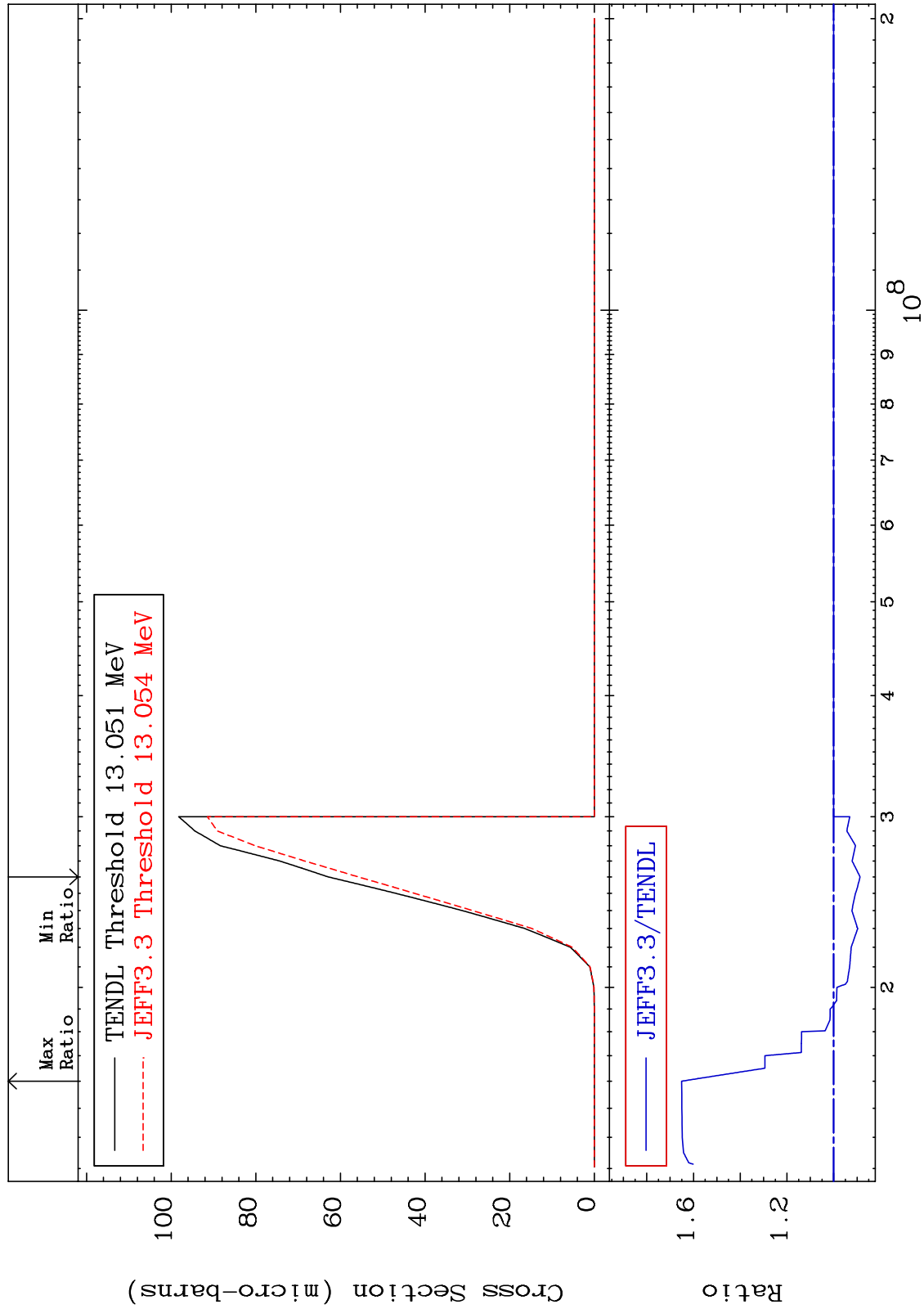
41-Nb-92

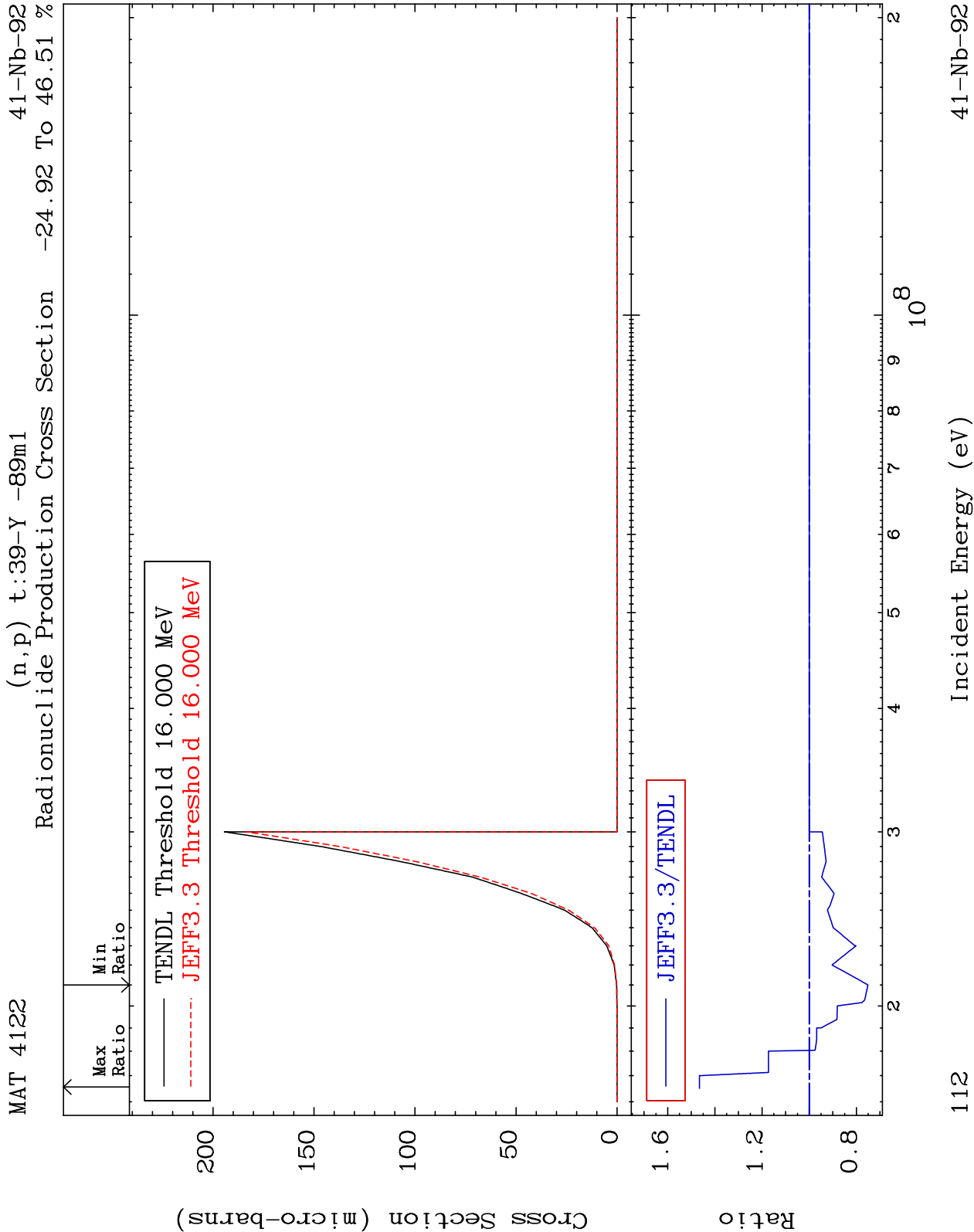






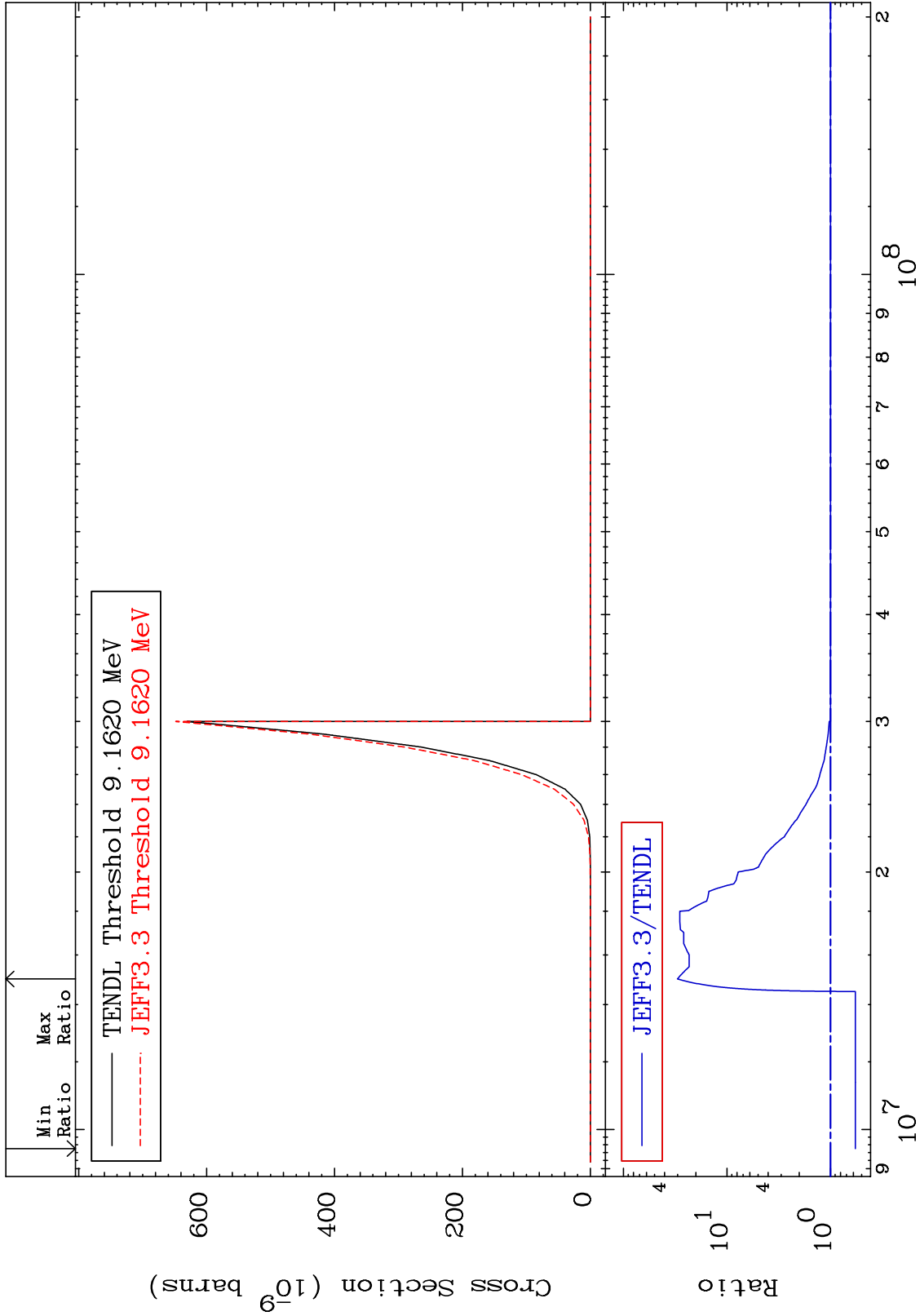






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(n,d) α :38-Sr-87g 41-Nb-92
Radionuclide Production Cross Section -42.55 To 2907. %



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Incident Energy (eV)

41-Nb-92

