

Program EVALPLOT
(Version 2018-1)

by

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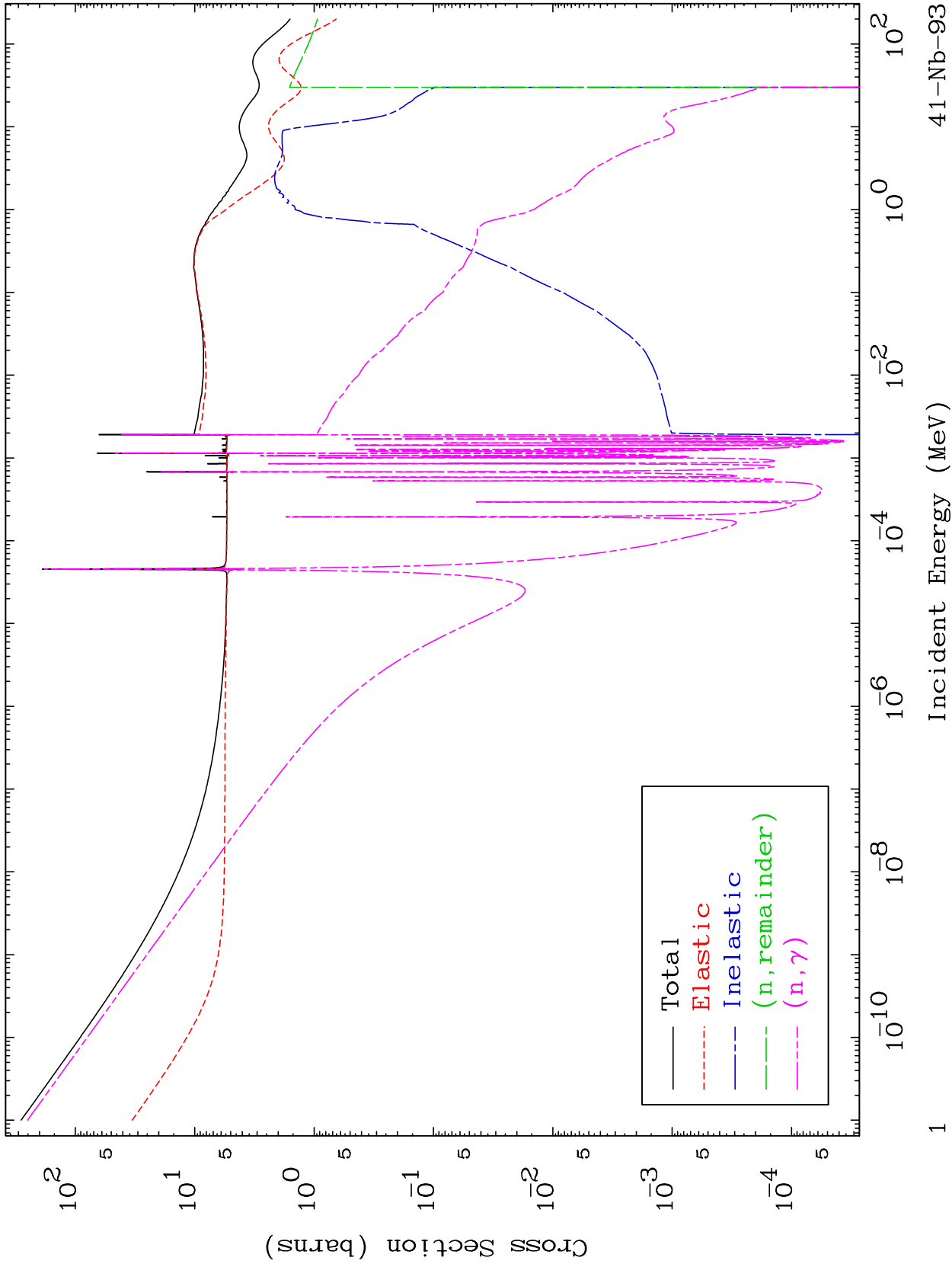
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Press Mouse Button to Start

MAT 4126

Major
293 Kelvin Cross Sections

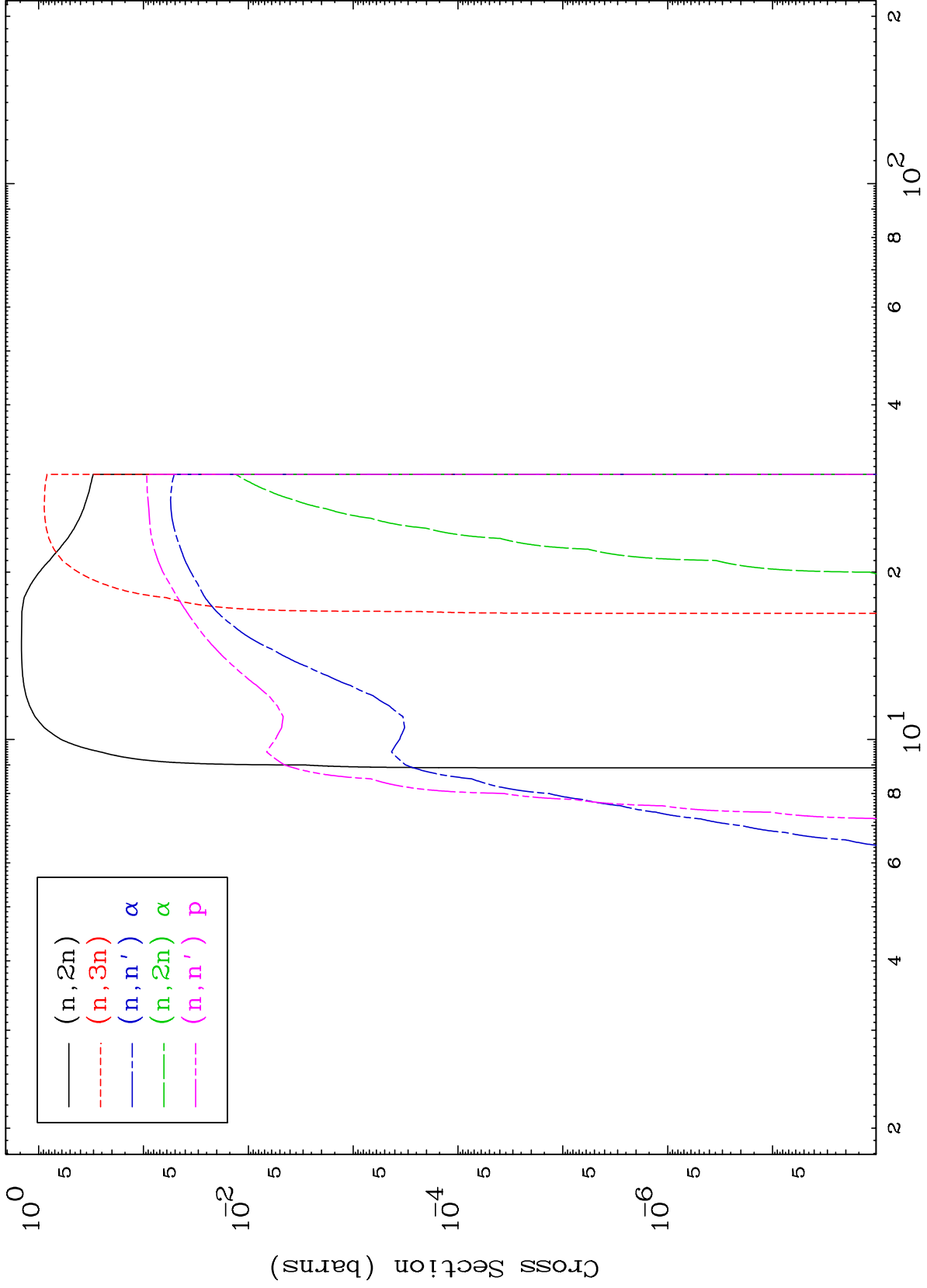
41-Nb-93



MAT 4126

Neutron Production
293 Kelvin Cross Sections

41-Nb-93



2

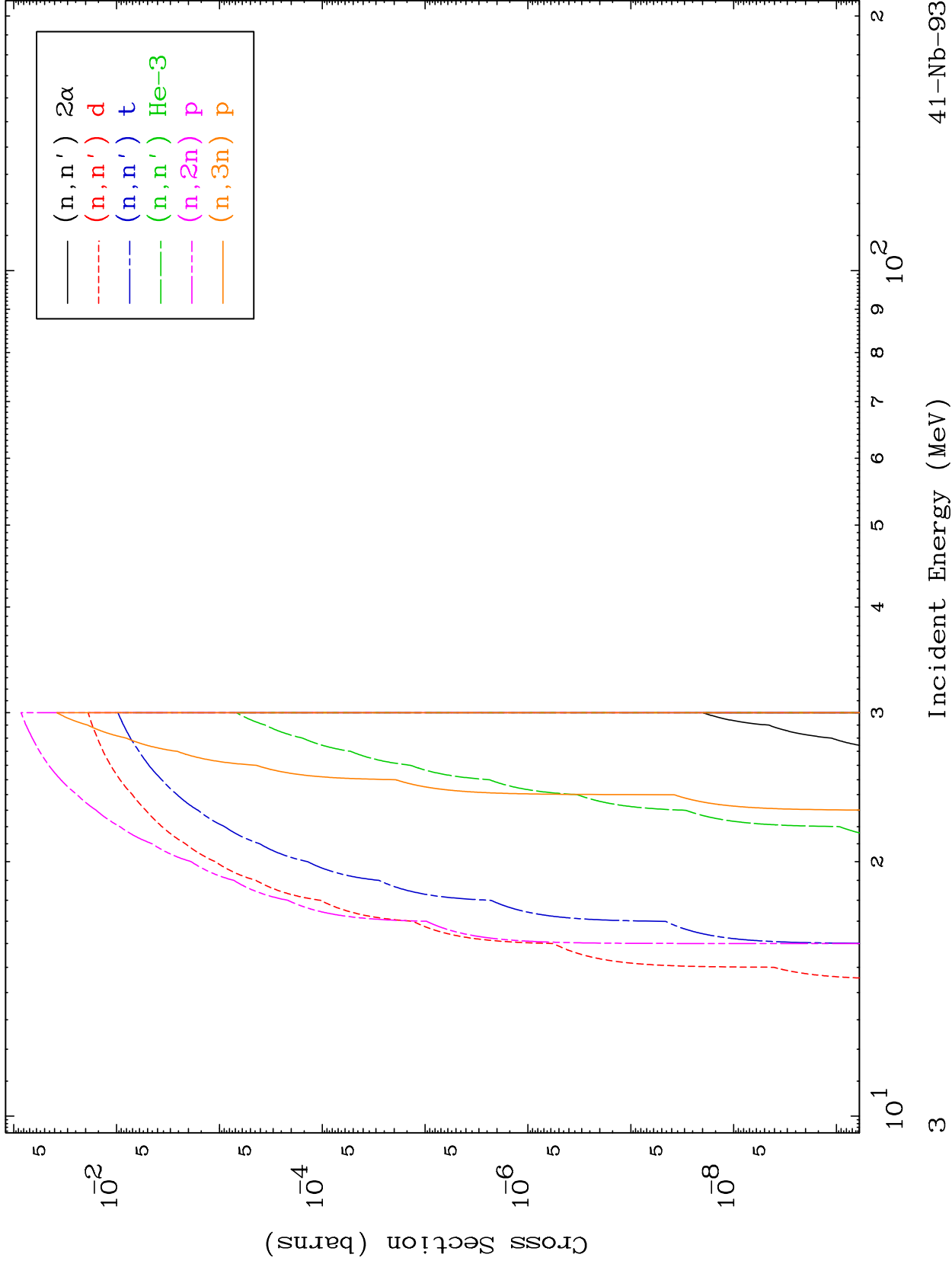
Incident Energy (MeV)

41-Nb-93

MAT 4126

Neutron Production
293 Kelvin Cross Sections

41-Nb-93

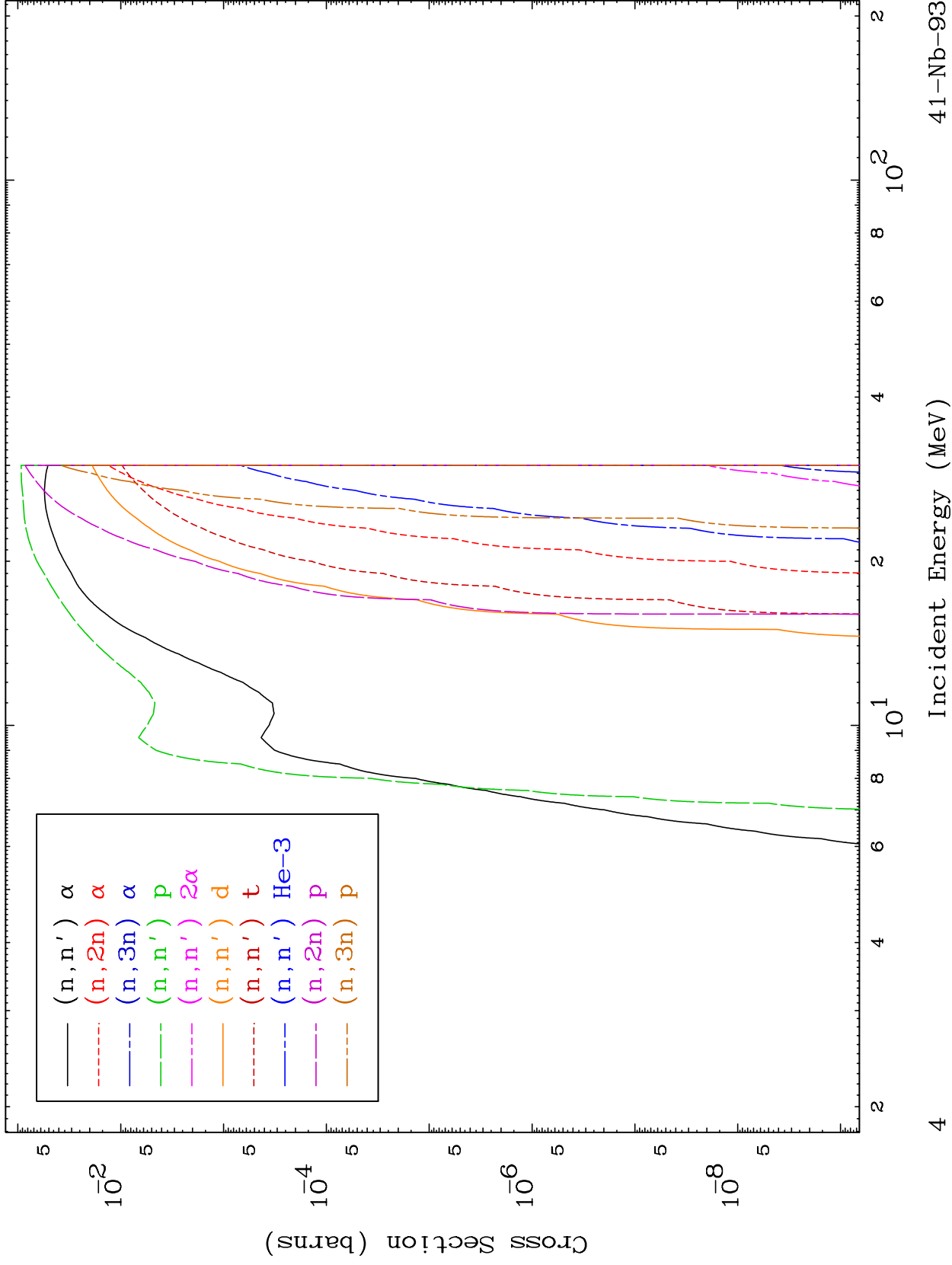


41-Nb-93

MAT 4126

Charged Particle
293 Kelvin Cross Sections

41-Nb-93



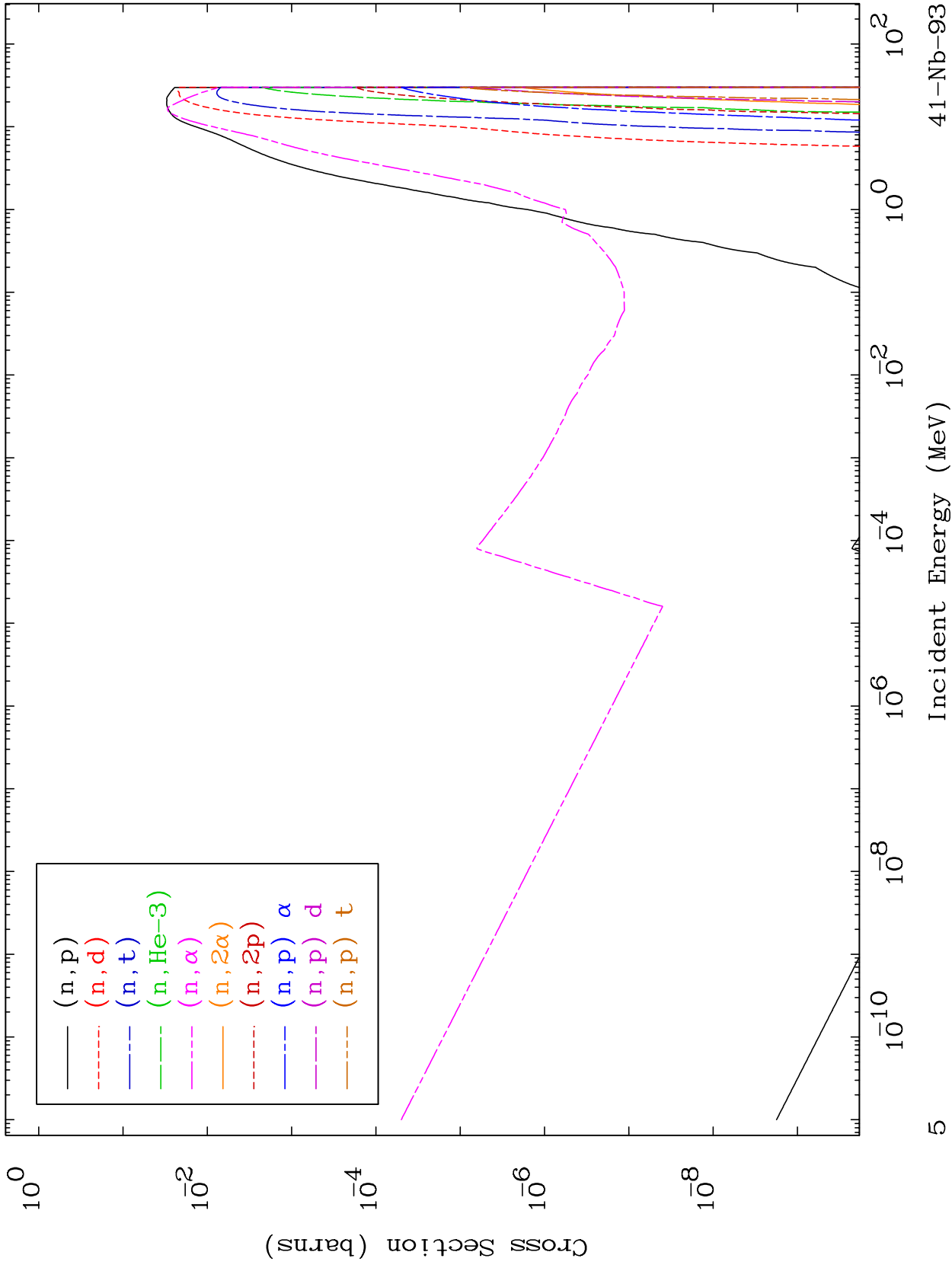
4

41-Nb-93

MAT 4126

Charged Particle
293 Kelvin Cross Sections

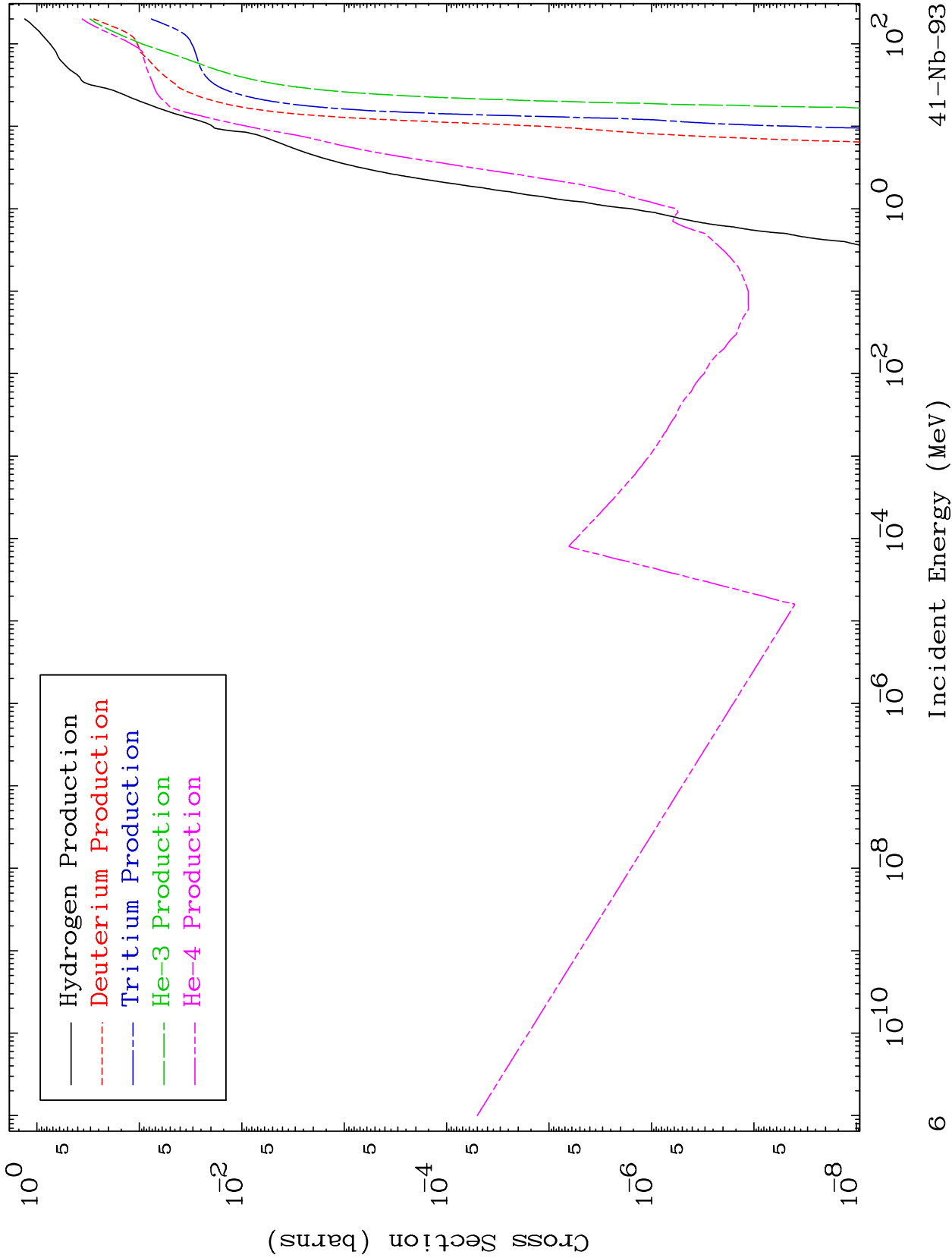
41-Nb-93



MAT 4126

Particle Production
293 Kelvin Cross Sections

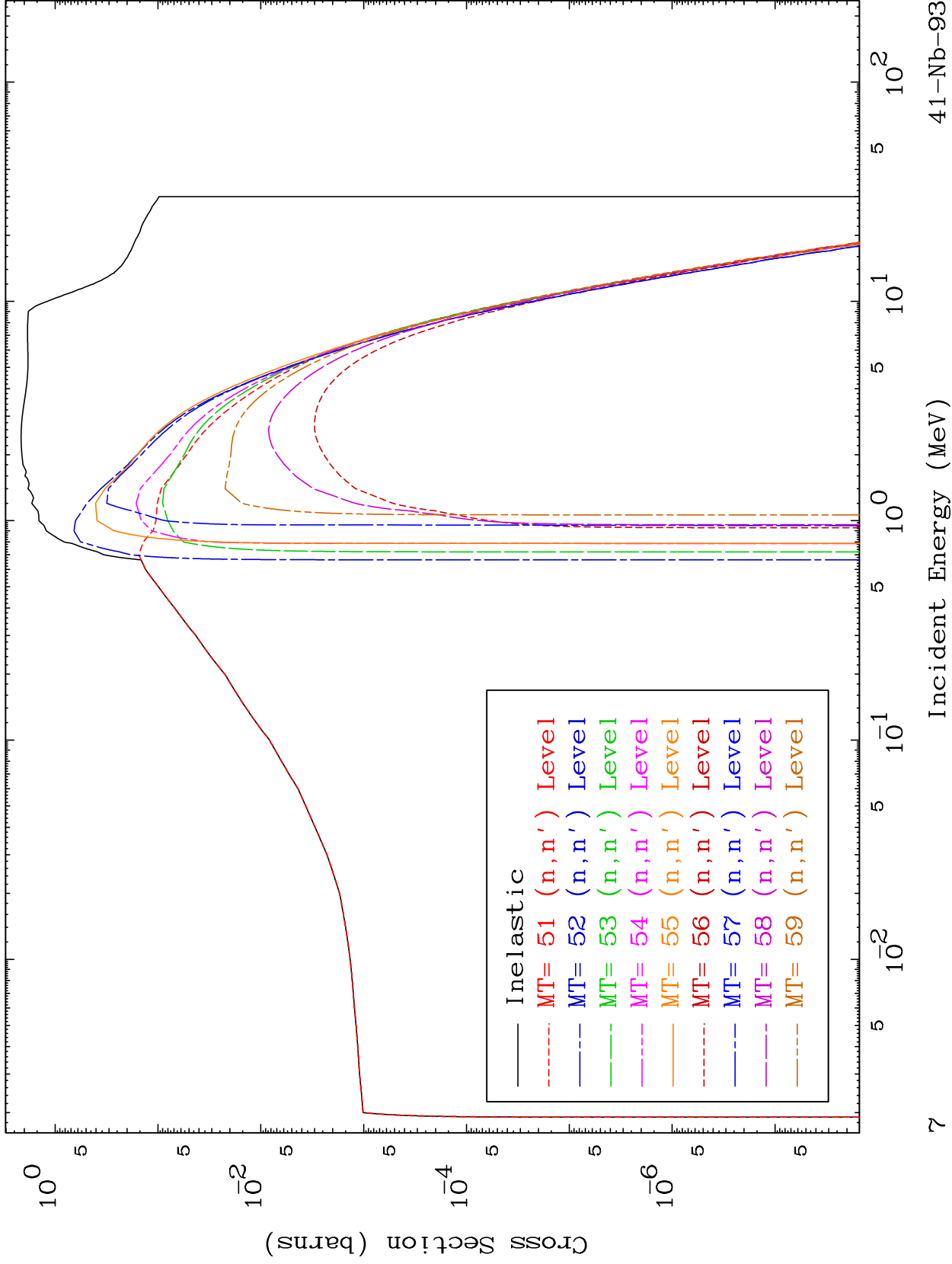
41-Nb-93



MAT 4126

293 Kelvin Cross Sections
(n,n') Level

41-Nb-93

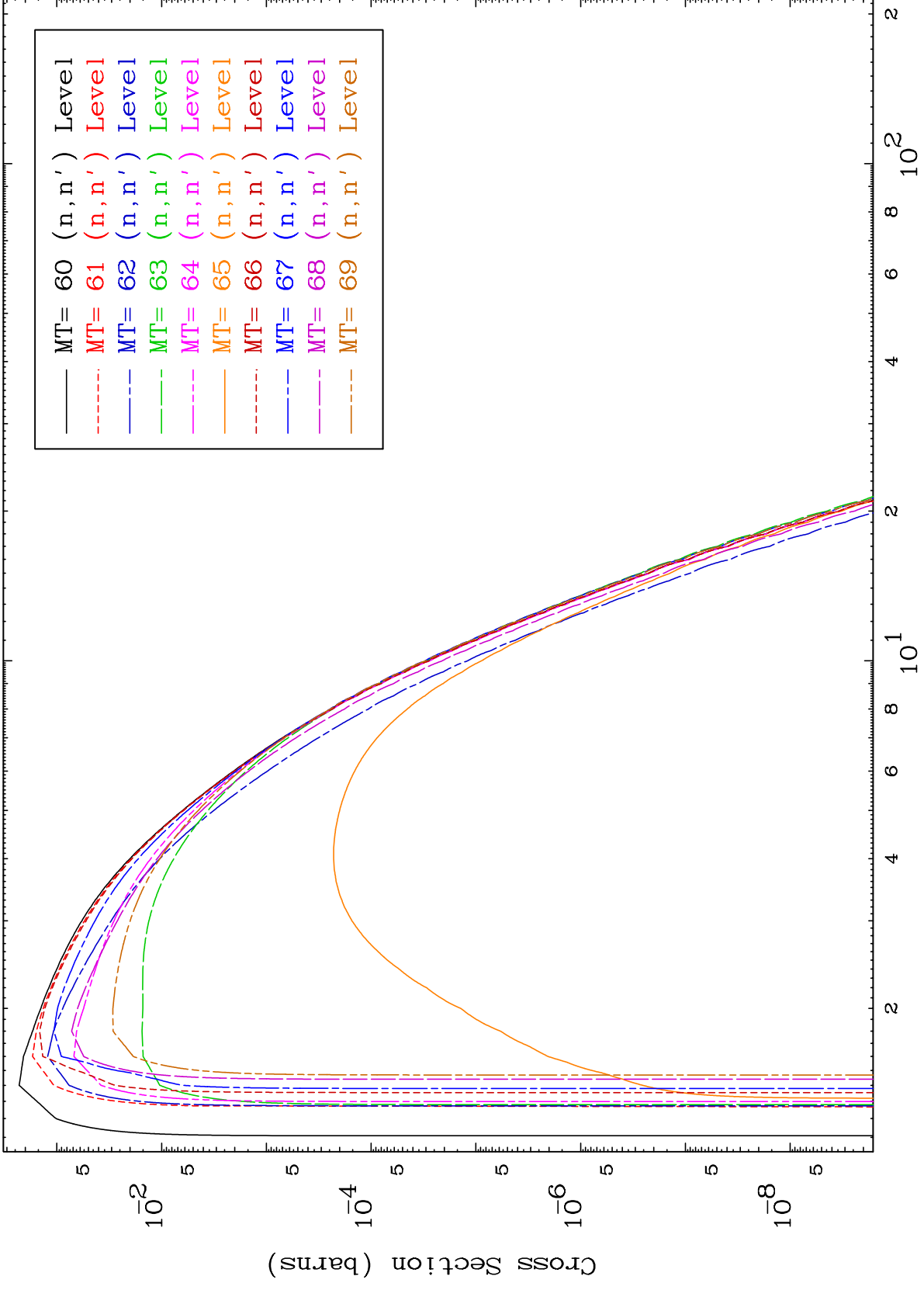


MAT 4126

(n,n') Level

41-Nb-93

293 Kelvin Cross Sections

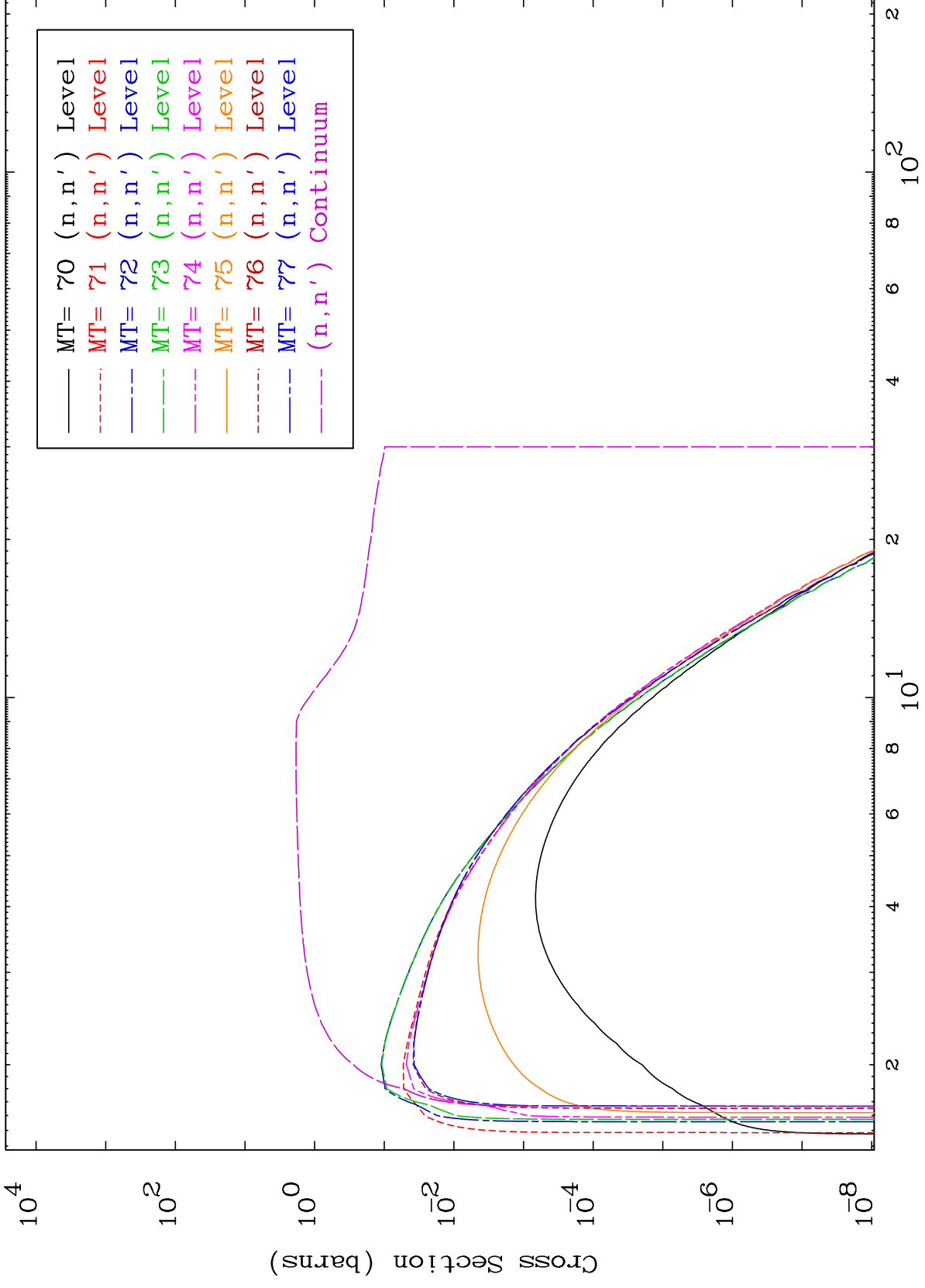


8

Incident Energy (MeV)

41-Nb-93

293 Kelvin Cross Sections

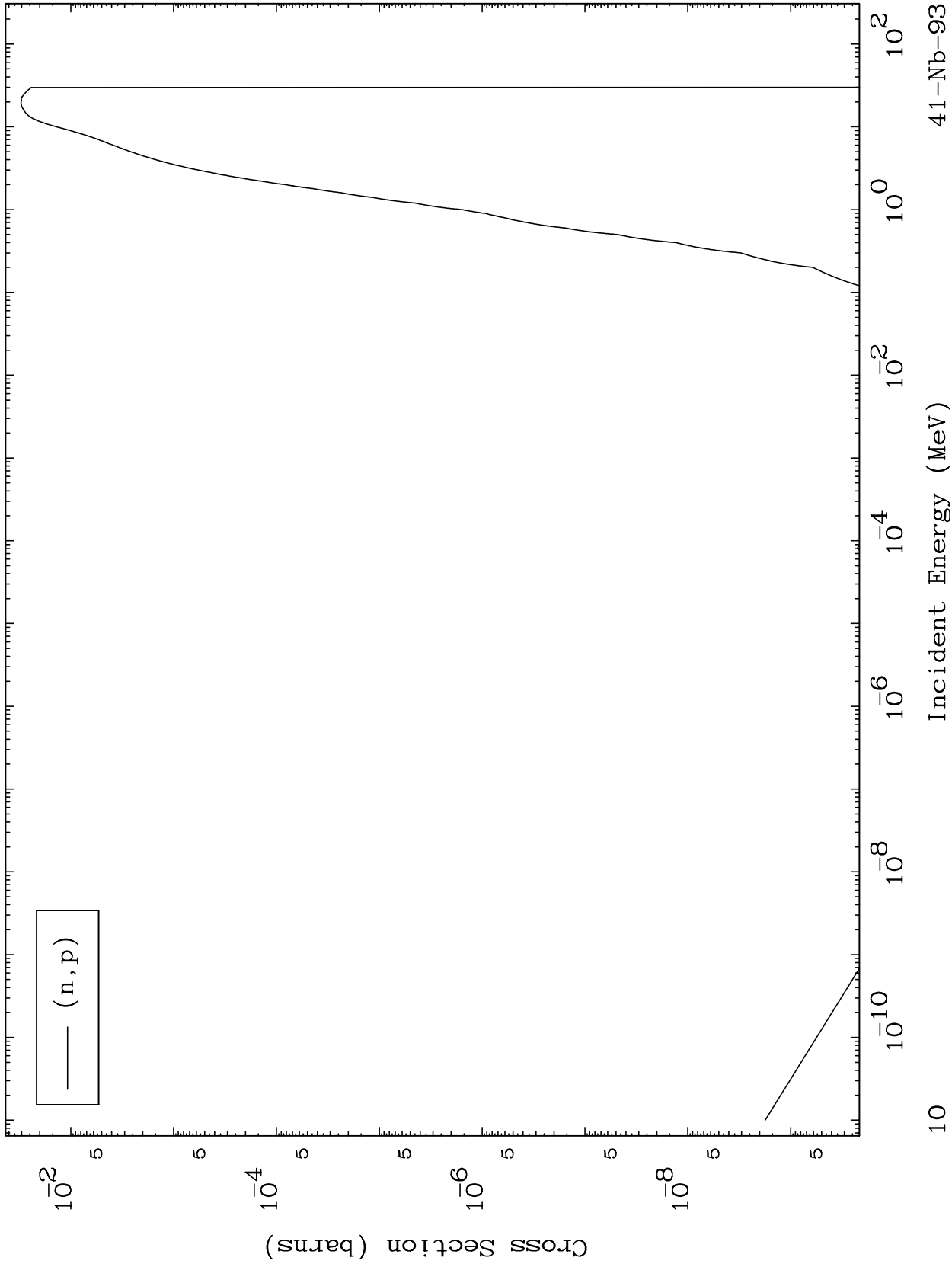


—	MT= 70	(n, n')	Level
- - -	MT= 71	(n, n')	Level
- . - .	MT= 72	(n, n')	Level
- - -	MT= 73	(n, n')	Level
- . - .	MT= 74	(n, n')	Level
—	MT= 75	(n, n')	Level
- - -	MT= 76	(n, n')	Level
- . - .	MT= 77	(n, n')	Level
- - -	(n, n')	Continuum	

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(n,p) Levels
293 Kelvin Cross Sections

41-Nb-93



10

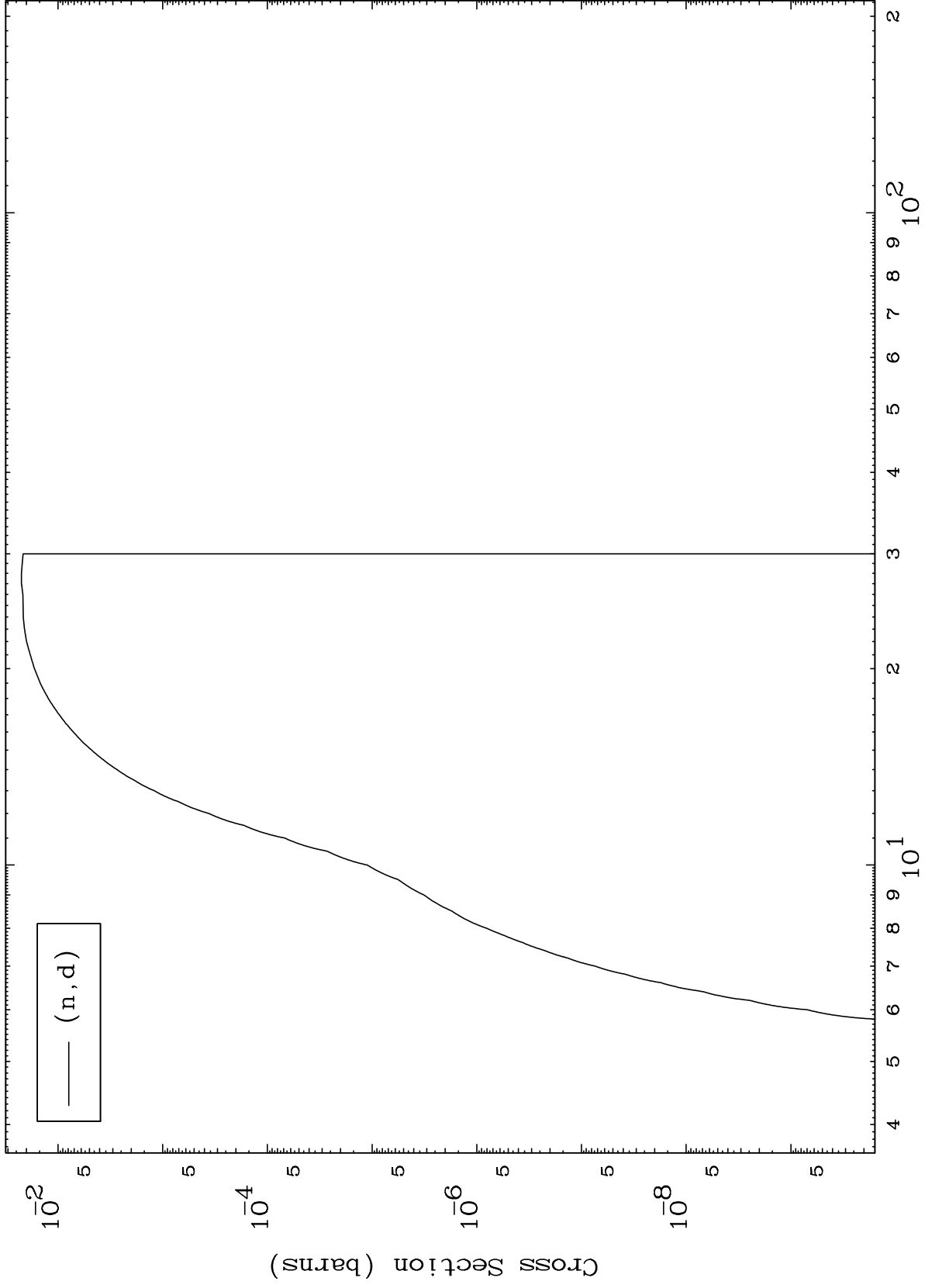
Incident Energy (MeV)

41-Nb-93

MAT 4126

(n,d) Levels
293 Kelvin Cross Sections

41-Nb-93



11

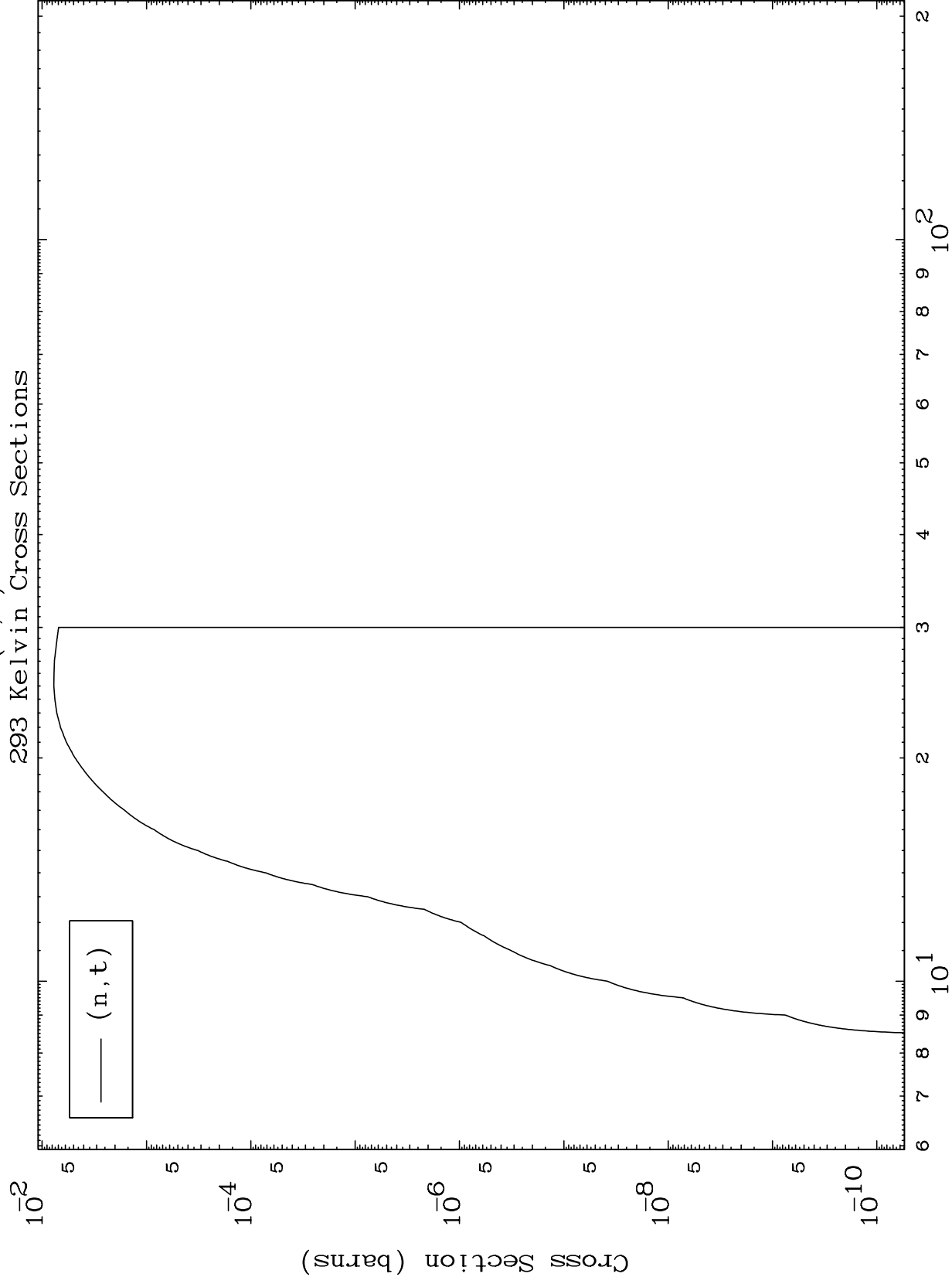
Incident Energy (MeV)

41-Nb-93

MAT 4126

(n,t) Levels
293 Kelvin Cross Sections

41-Nb-93



12

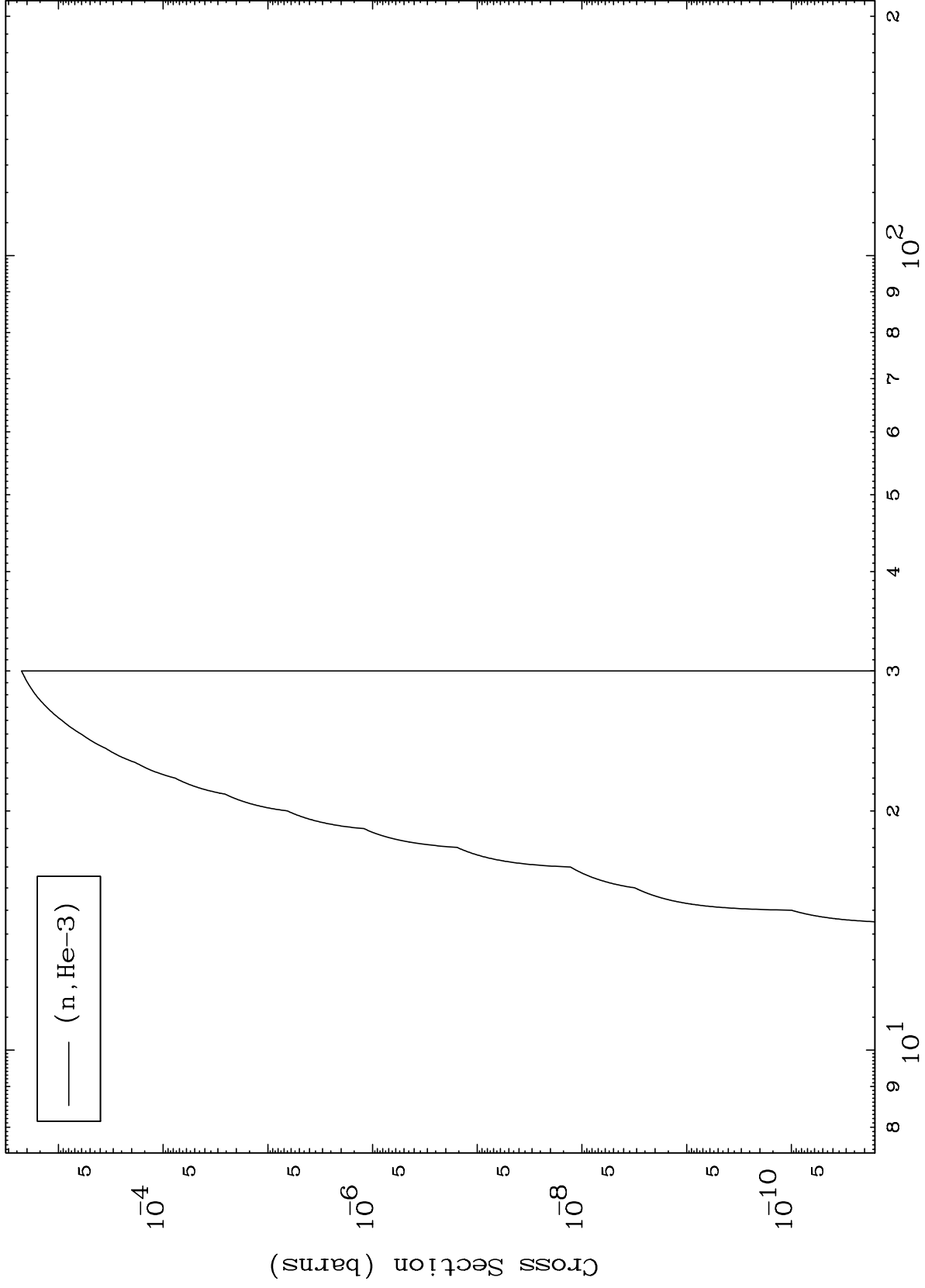
Incident Energy (MeV)

41-Nb-93

MAT 4126

(n,He3) Levels
293 Kelvin Cross Sections

41-Nb-93



13

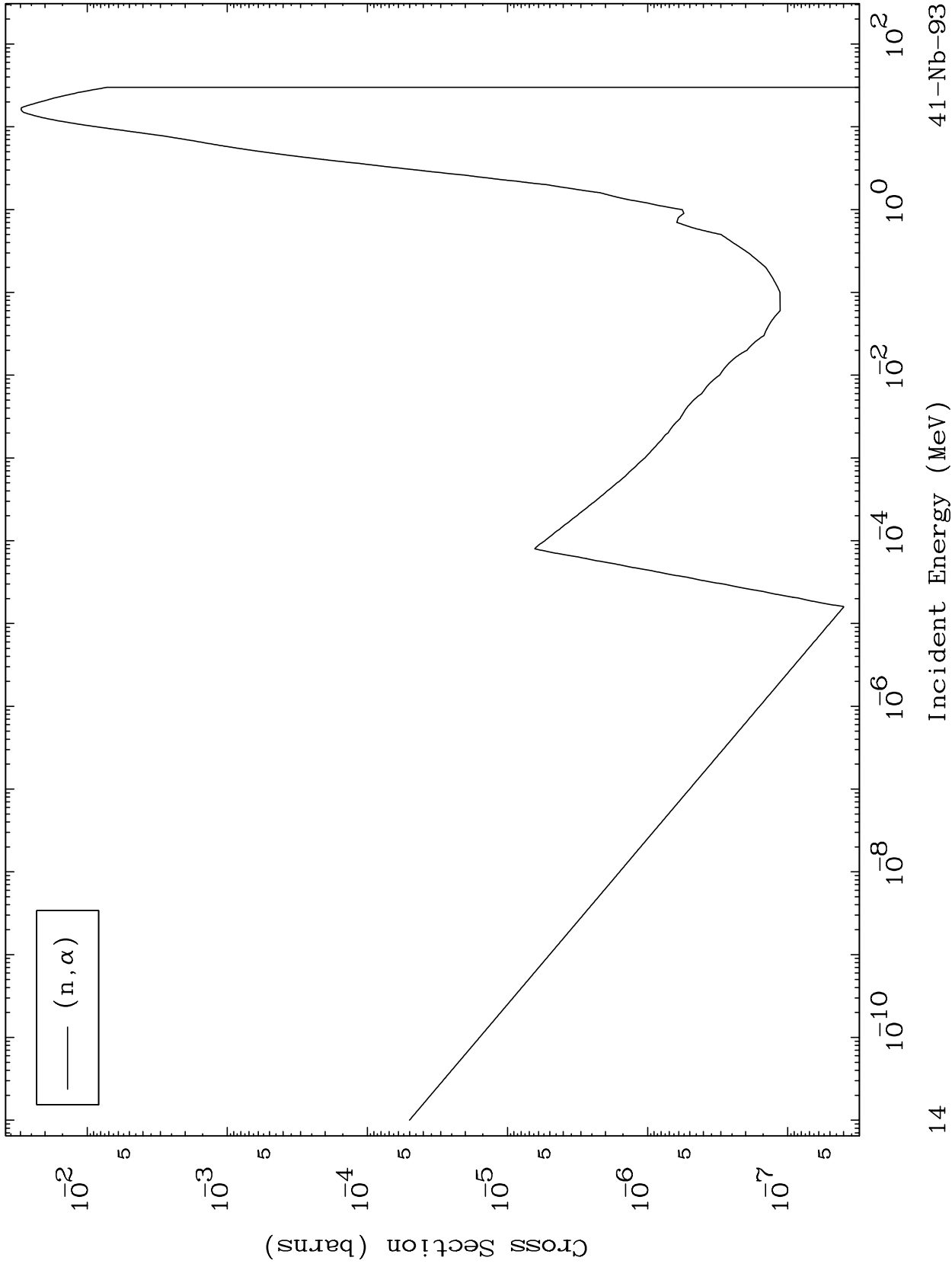
Incident Energy (MeV)

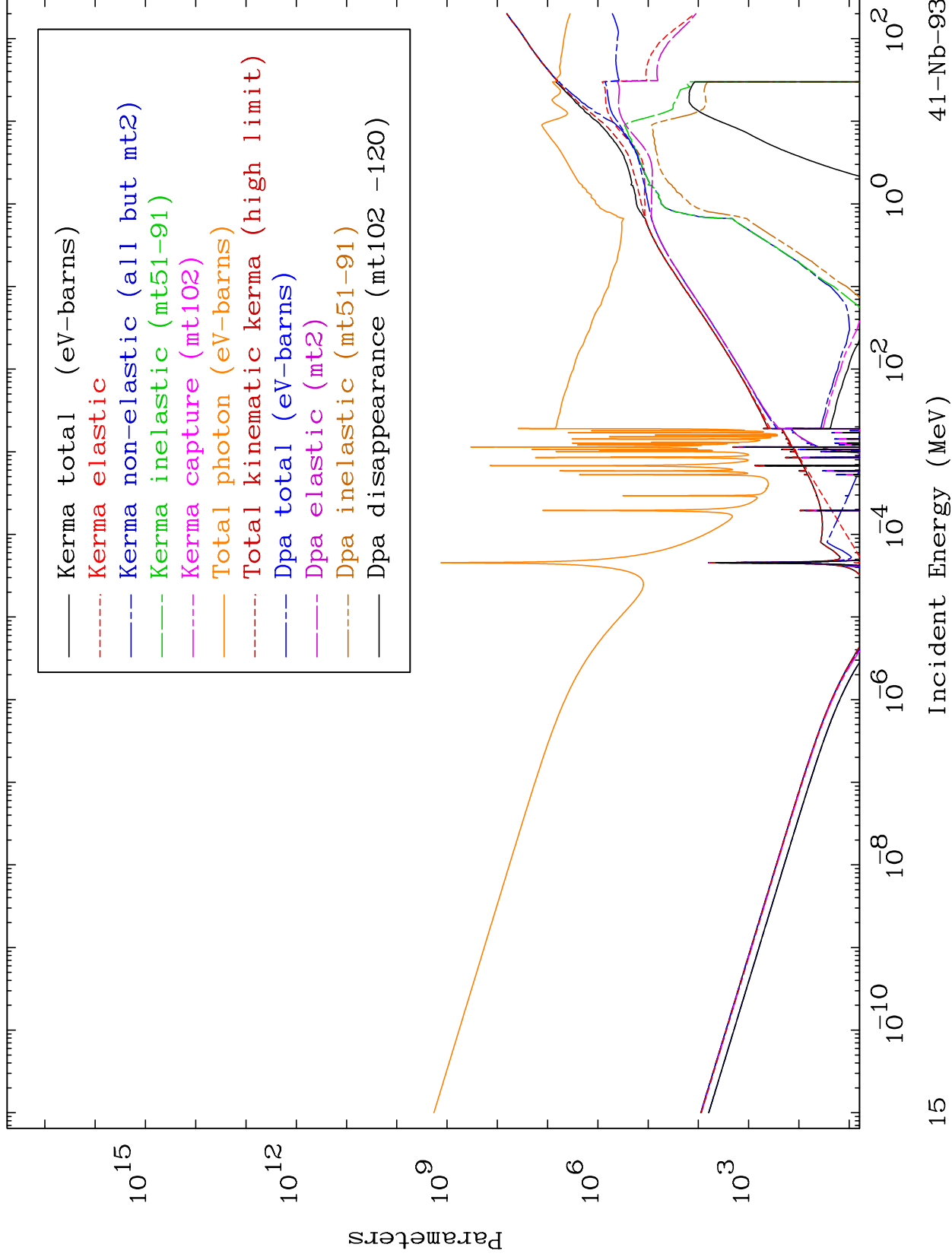
41-Nb-93

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(n, α) Levels
293 Kelvin Cross Sections

41-Nb-93

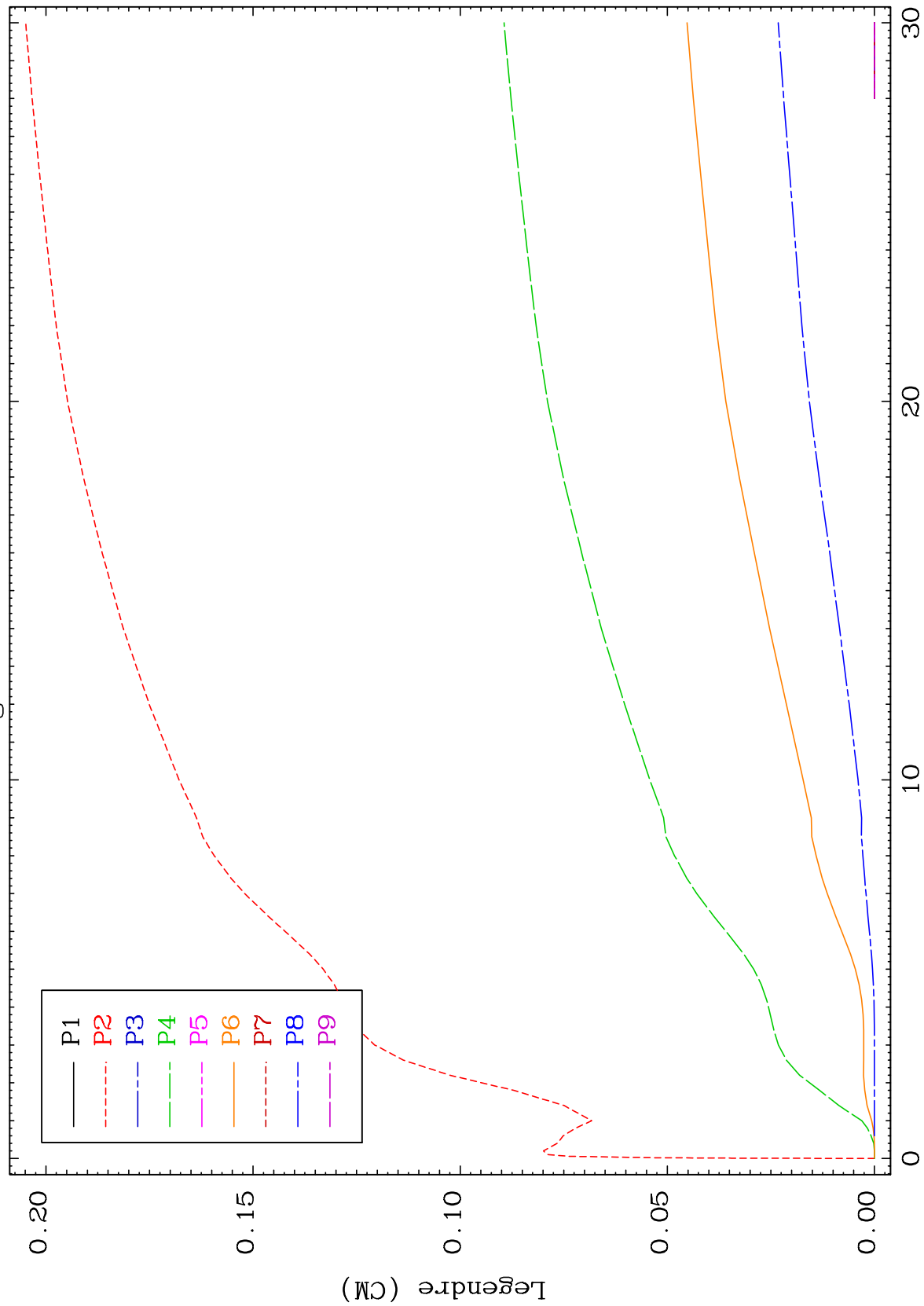




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Elastic Legendre Coefficients

41-Nb-93



16

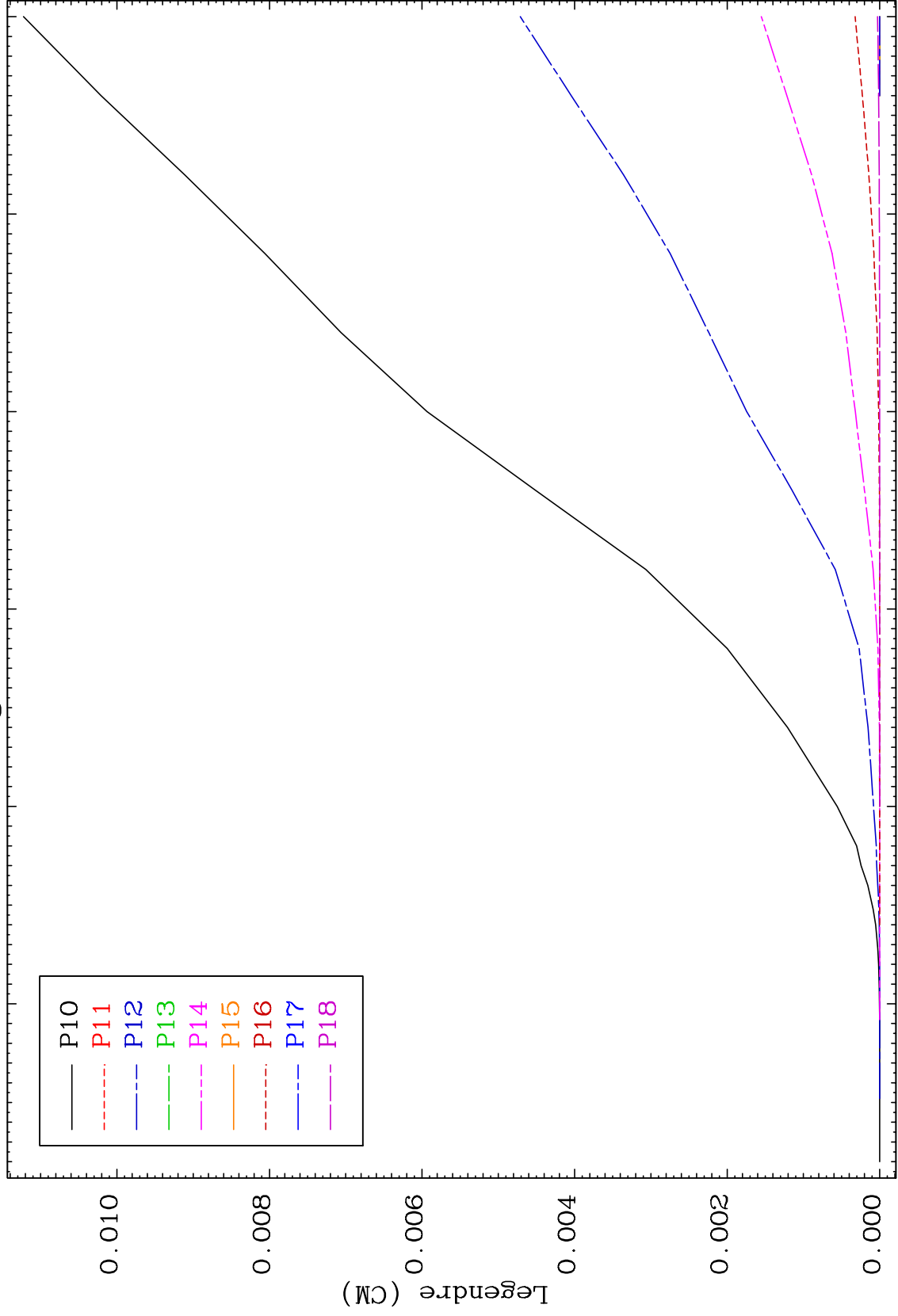
Incident Energy (MeV)

41-Nb-93

MAT 4126

Elastic Legendre Coefficients

41-Nb-93



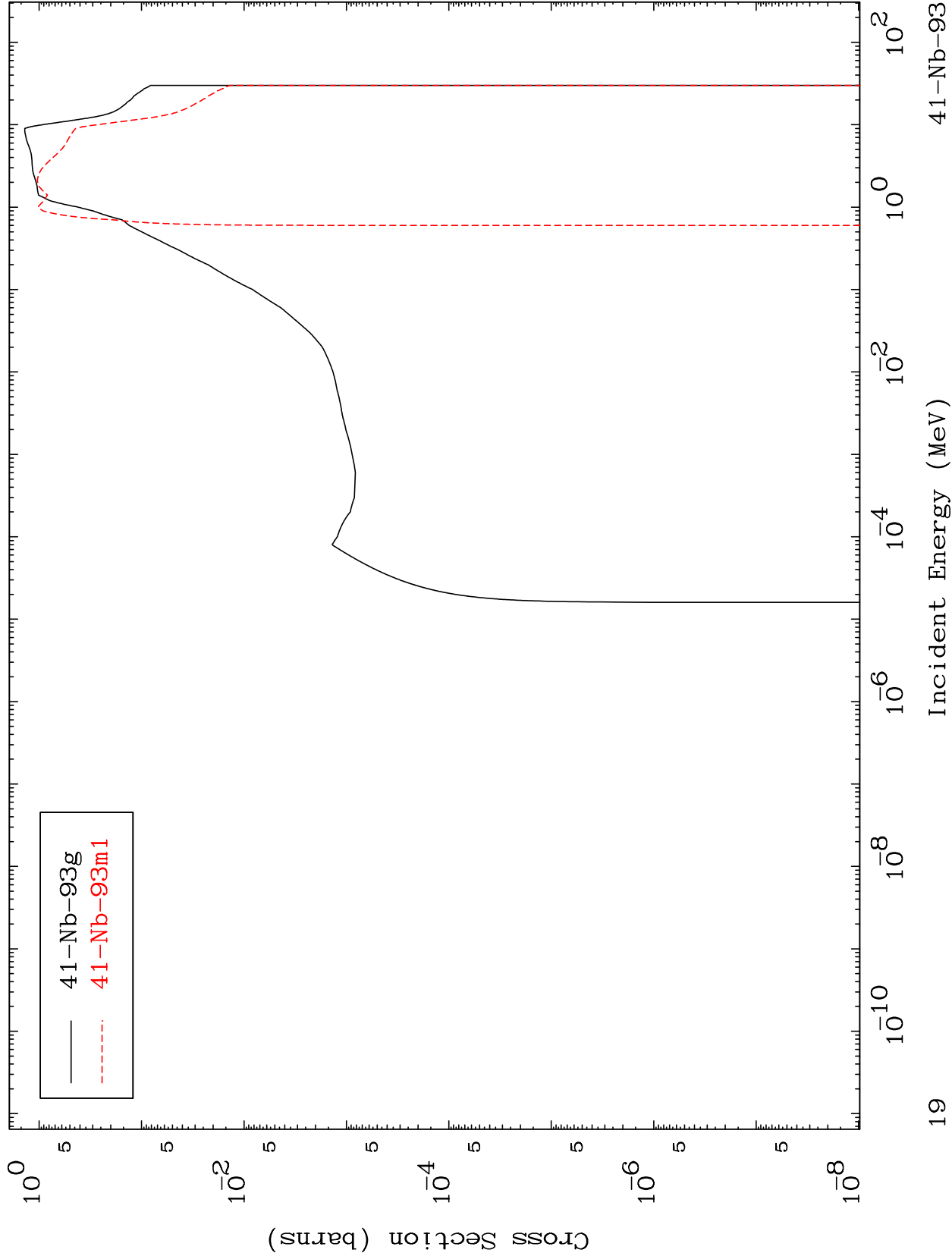
17

41-Nb-93

MAT 4126

41-Nb-93

Inelastic
Radionuclide Production Cross Section

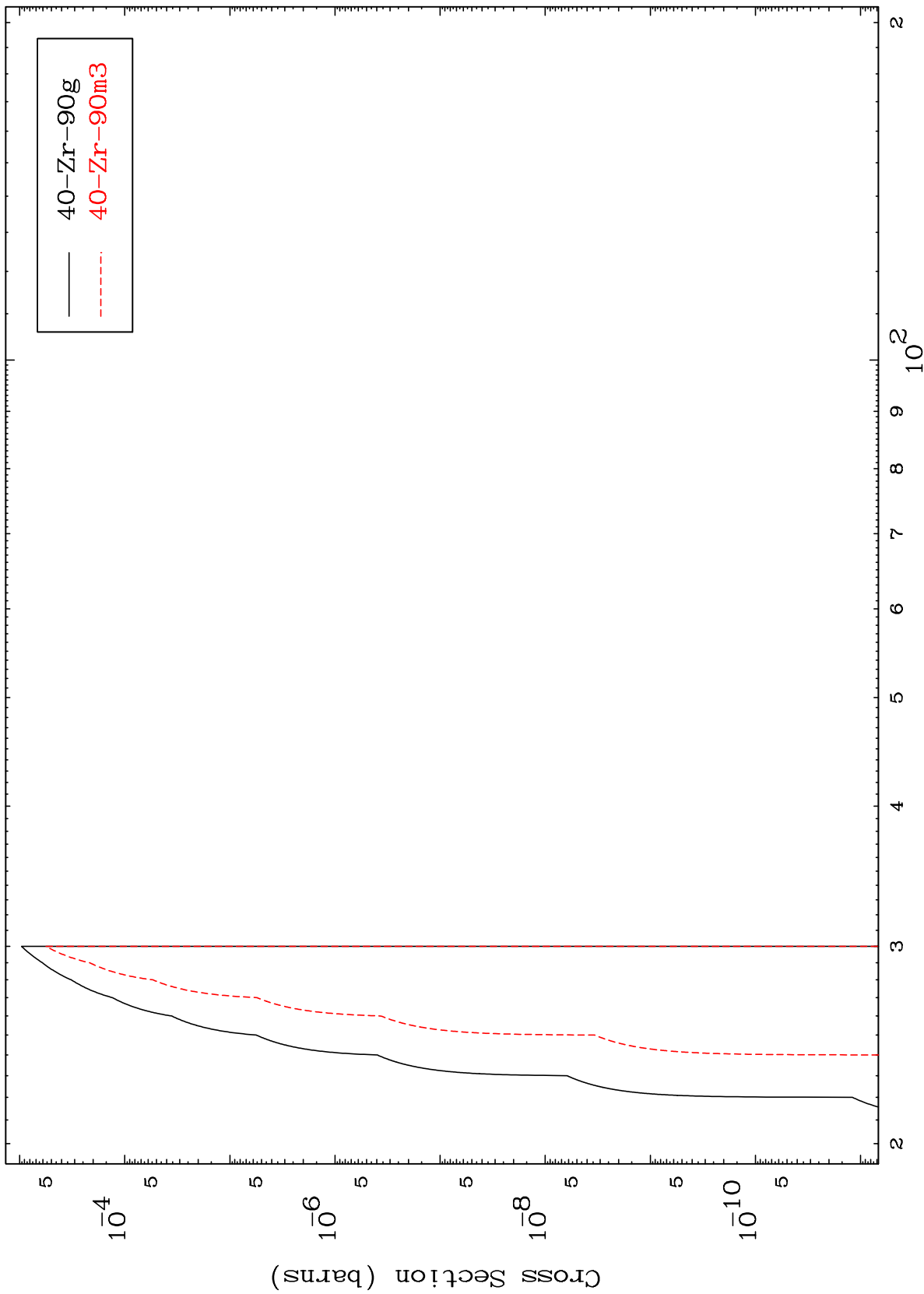


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(n,2n) d

41-Nb-93

Radionuclide Production Cross Section



20

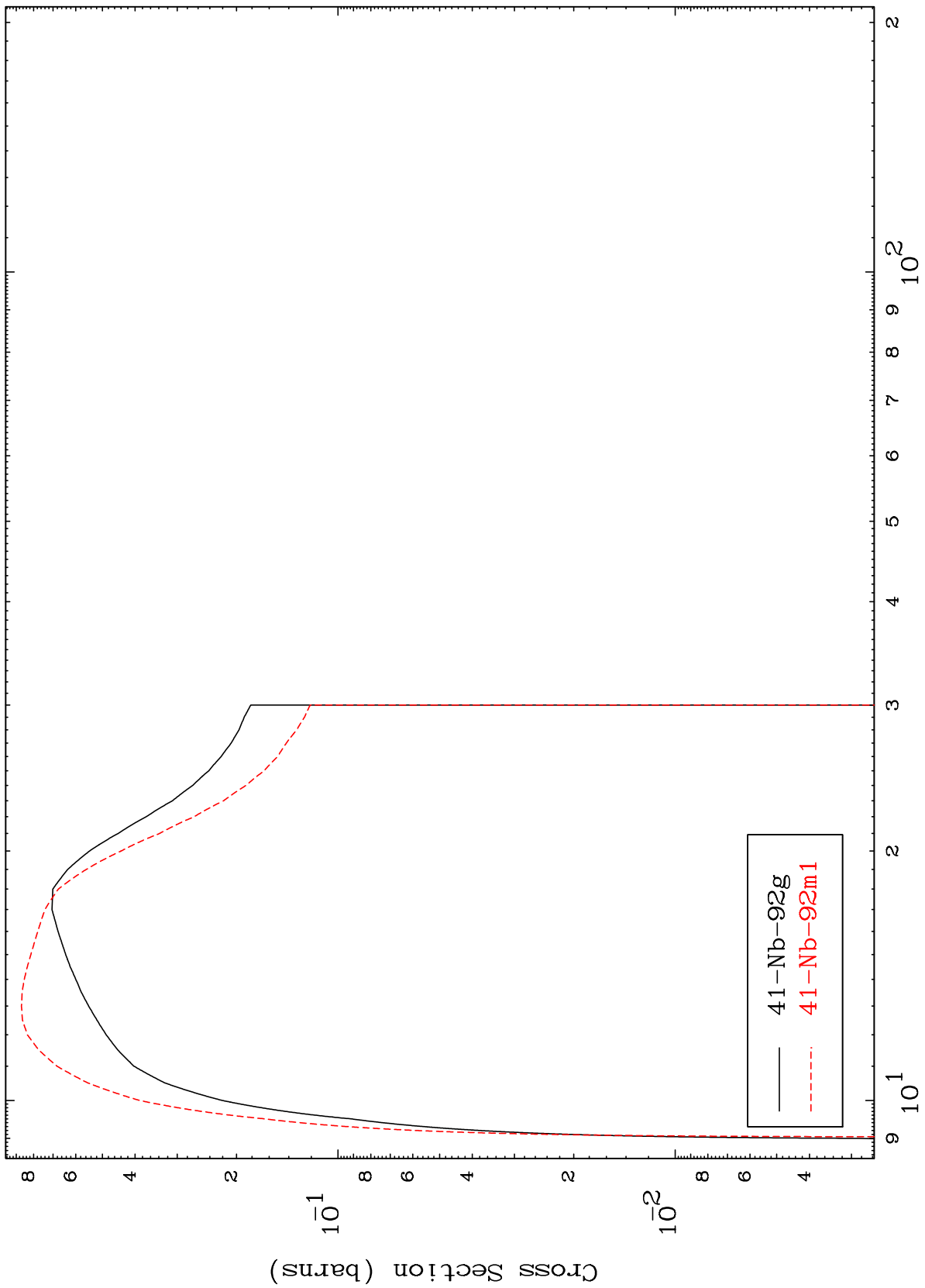
Incident Energy (MeV)

41-Nb-93

MAT 4126

41-Nb-93

(n,2n)
Radionuclide Production Cross Section



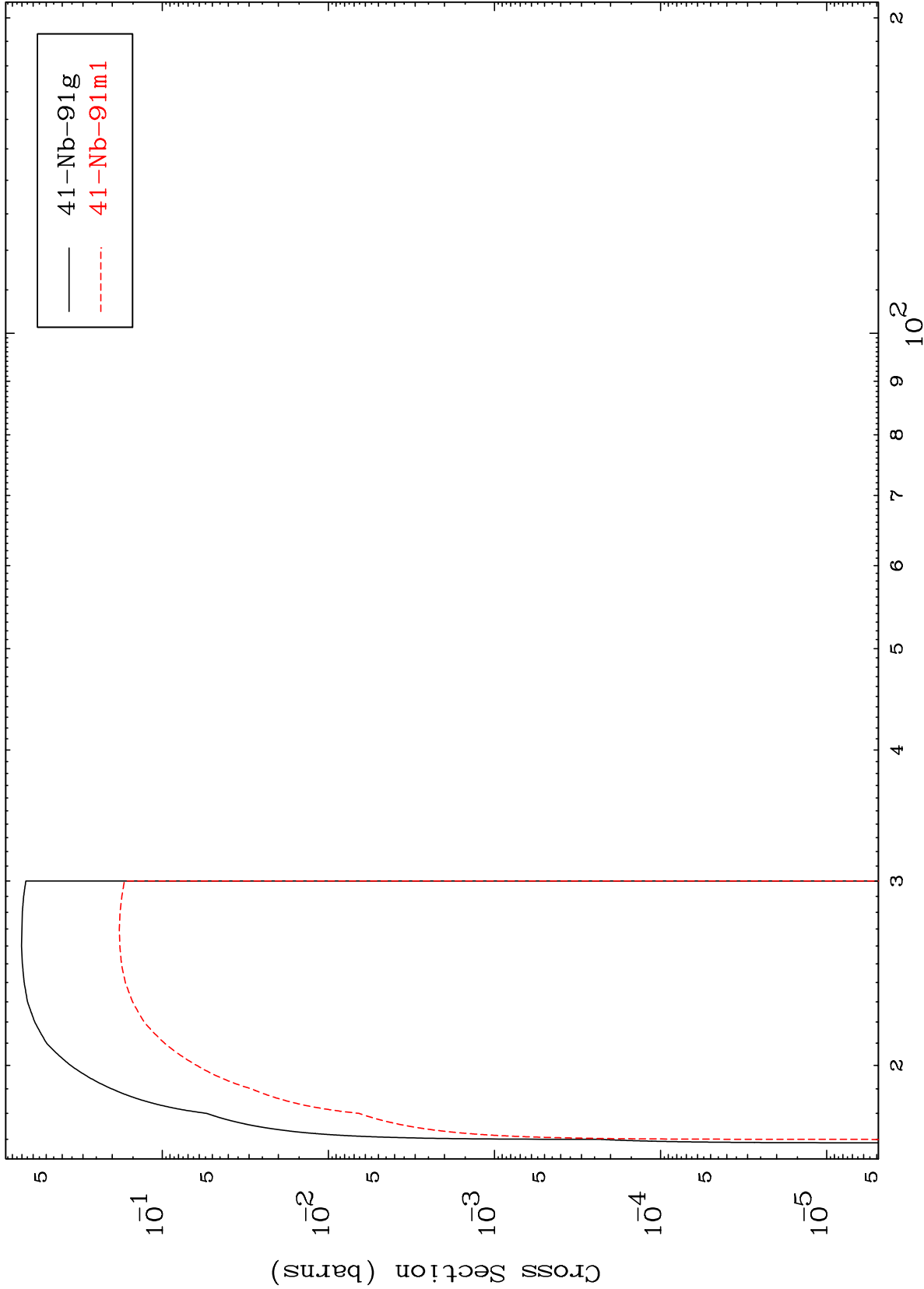
41-Nb-92g
41-Nb-92m1

41-Nb-93

Incident Energy (MeV)

21

Radionuclide Production Cross Section

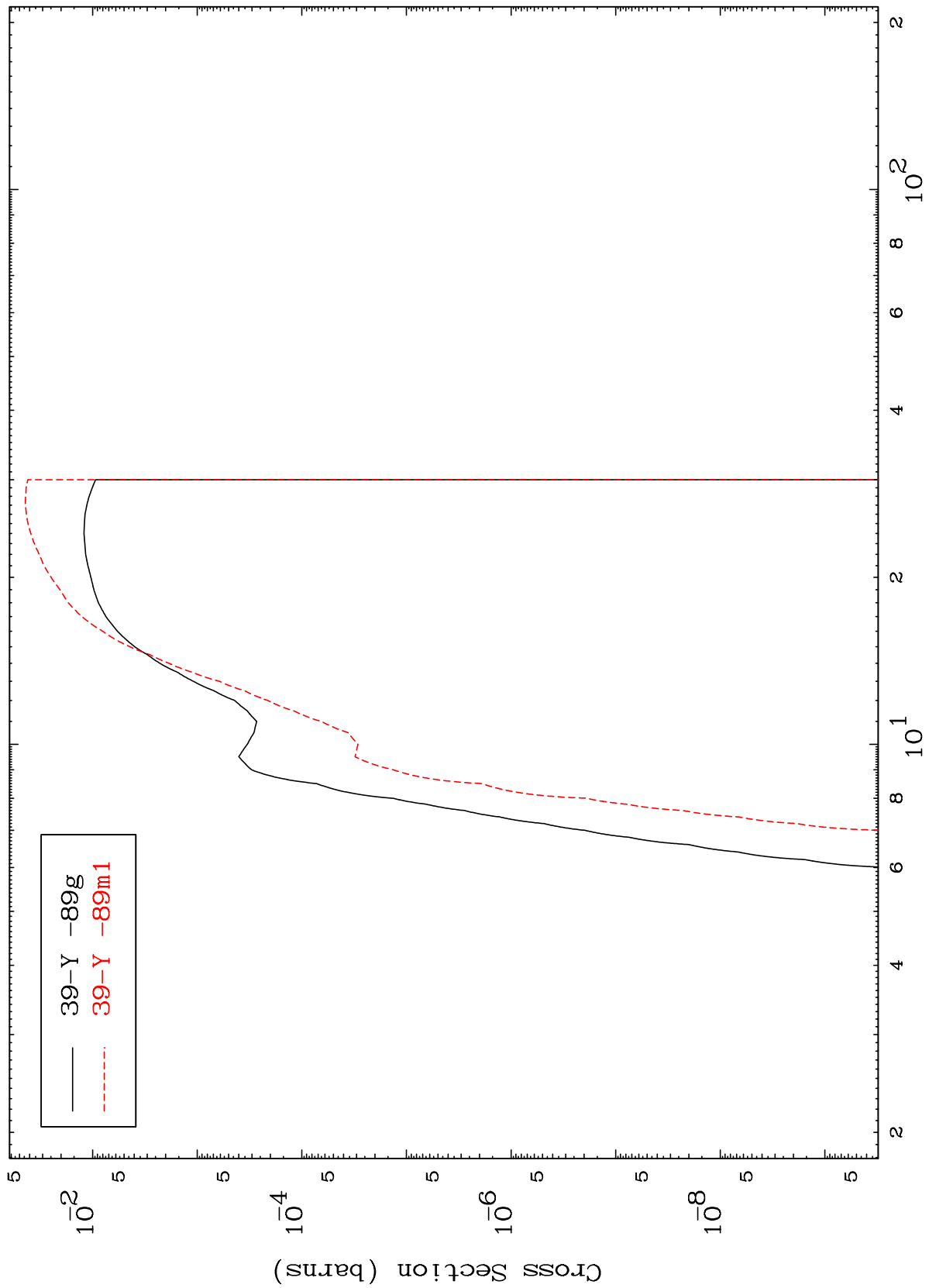


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$(n, n') \alpha$

41-Nb-93

Radionuclide Production Cross Section

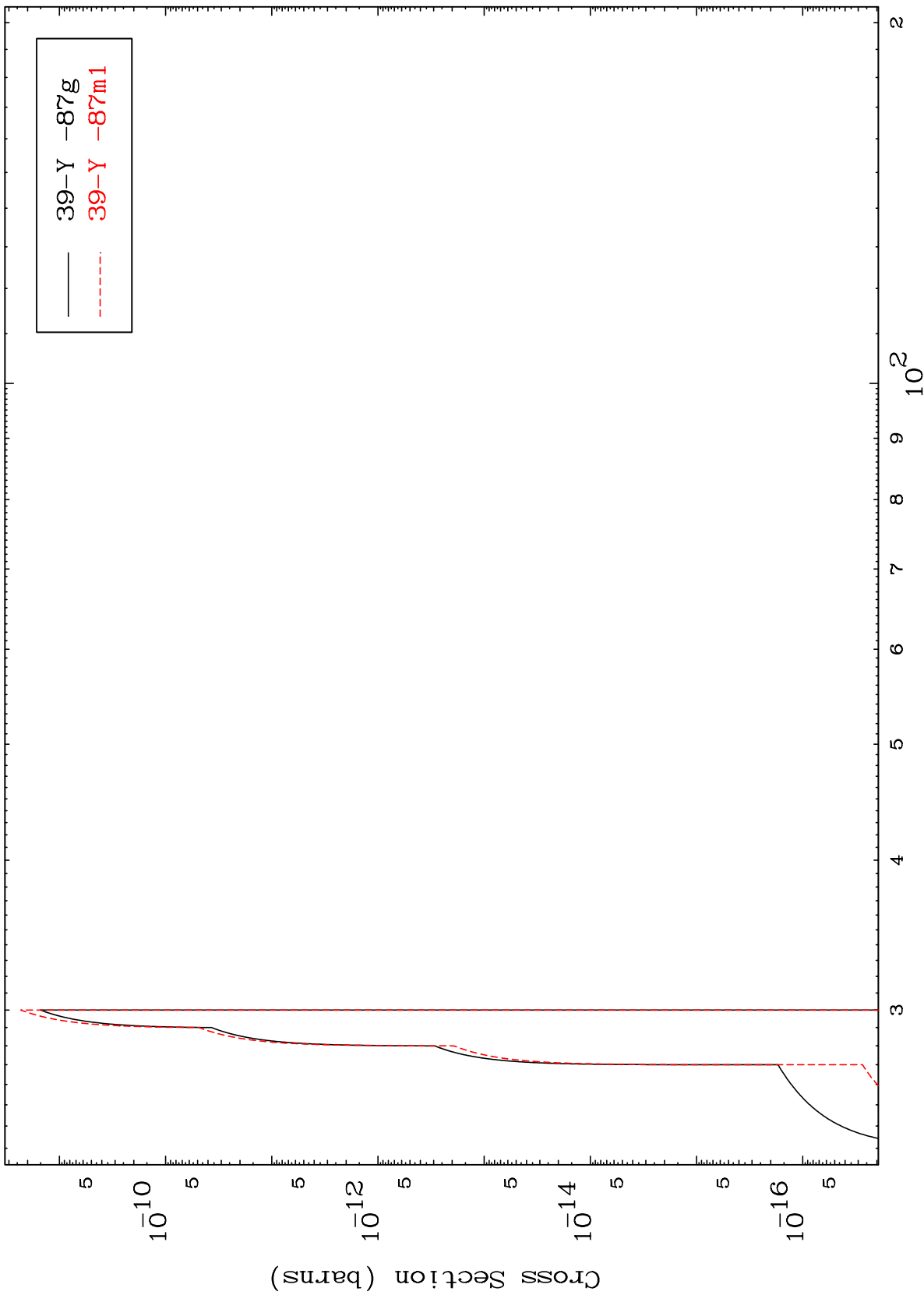


23

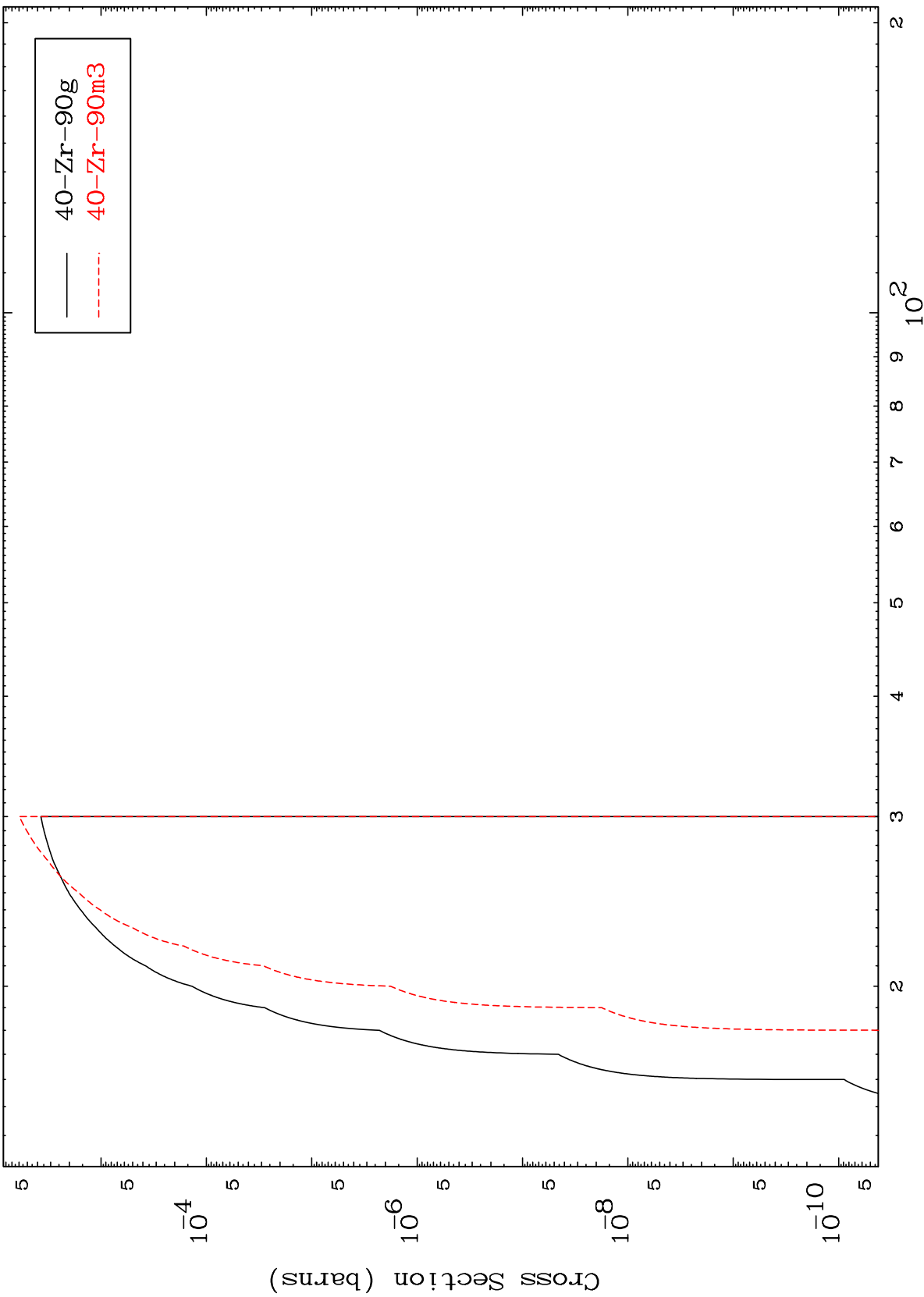
Incident Energy (MeV)

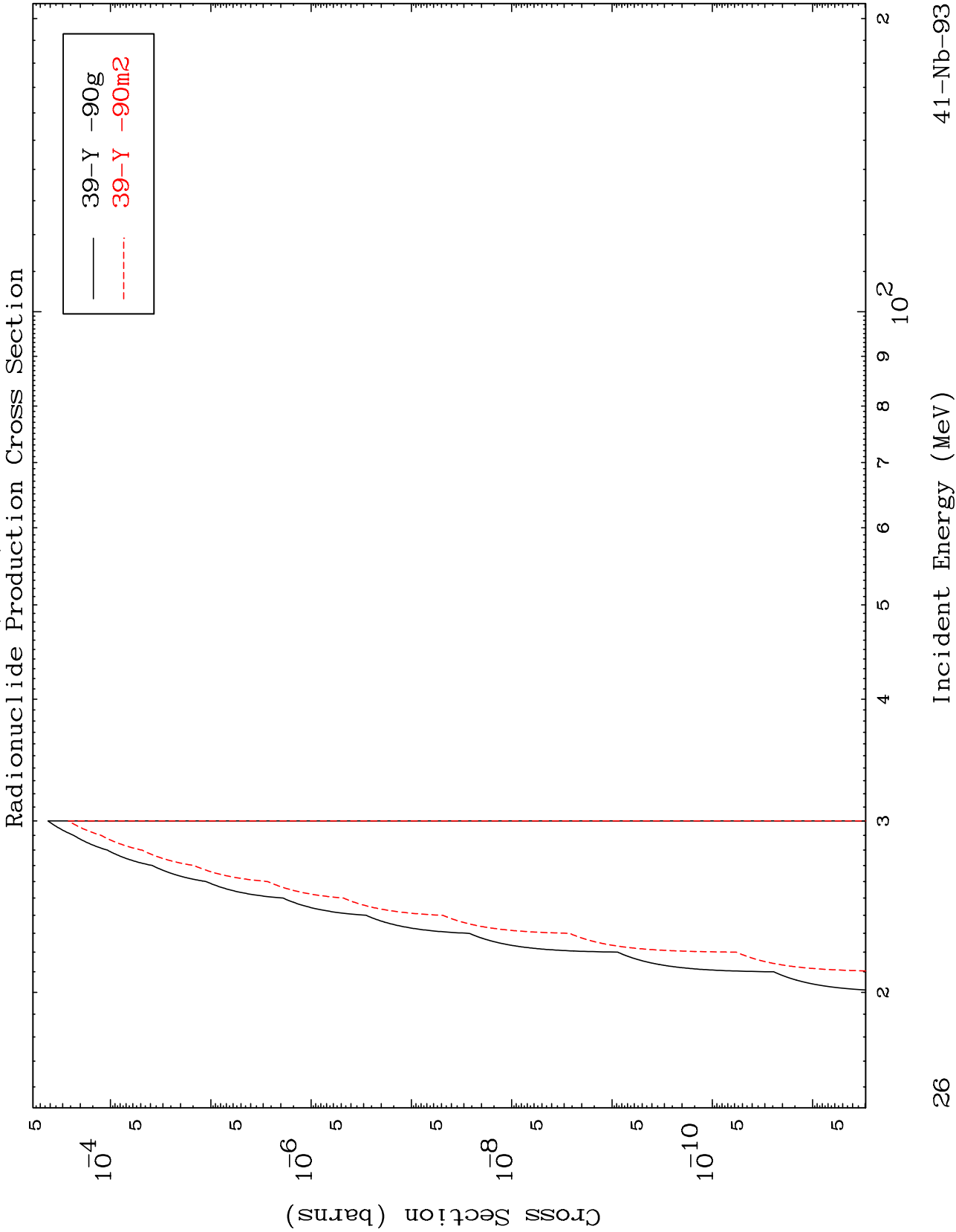
41-Nb-93

Radionuclide Production Cross Section

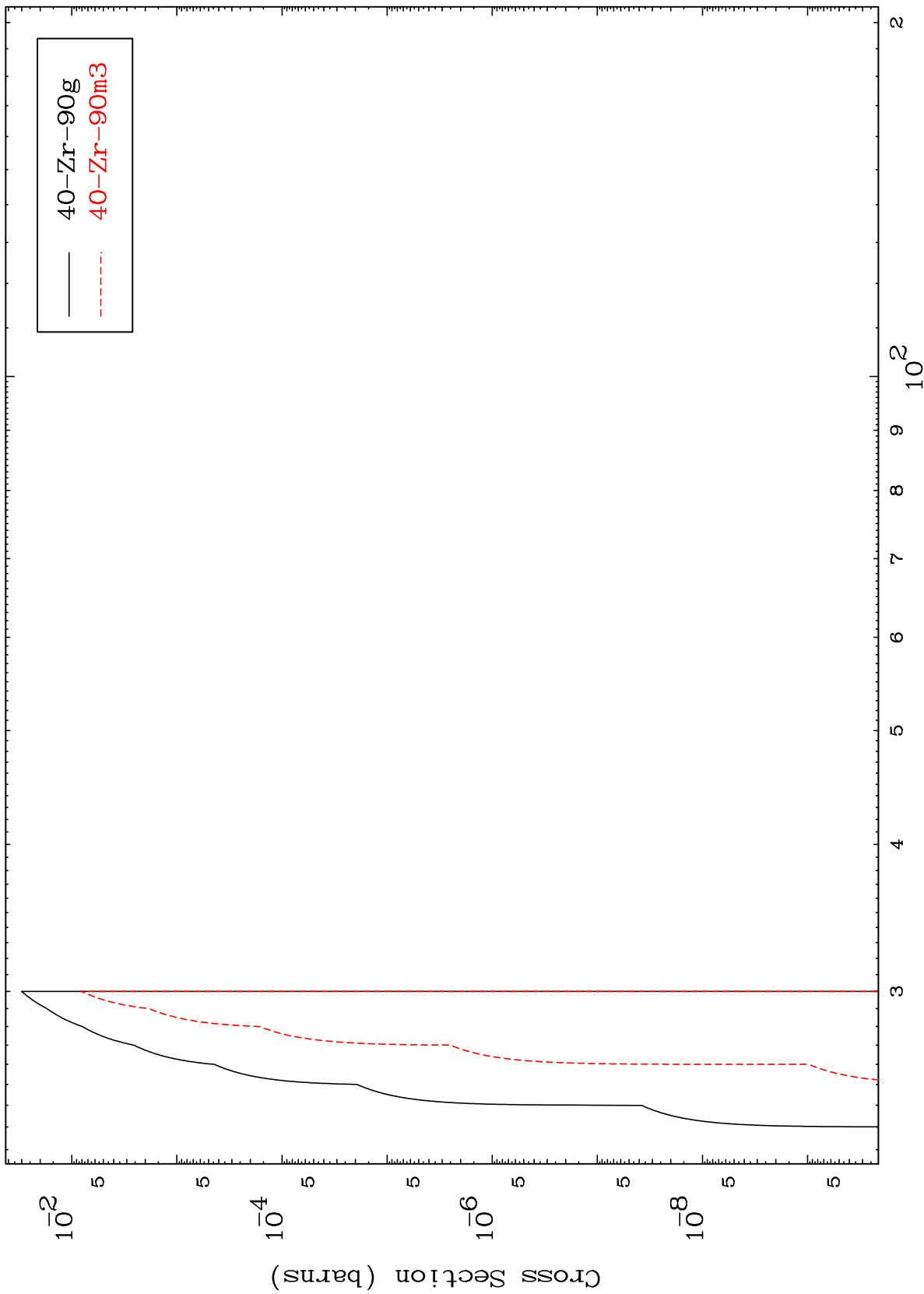


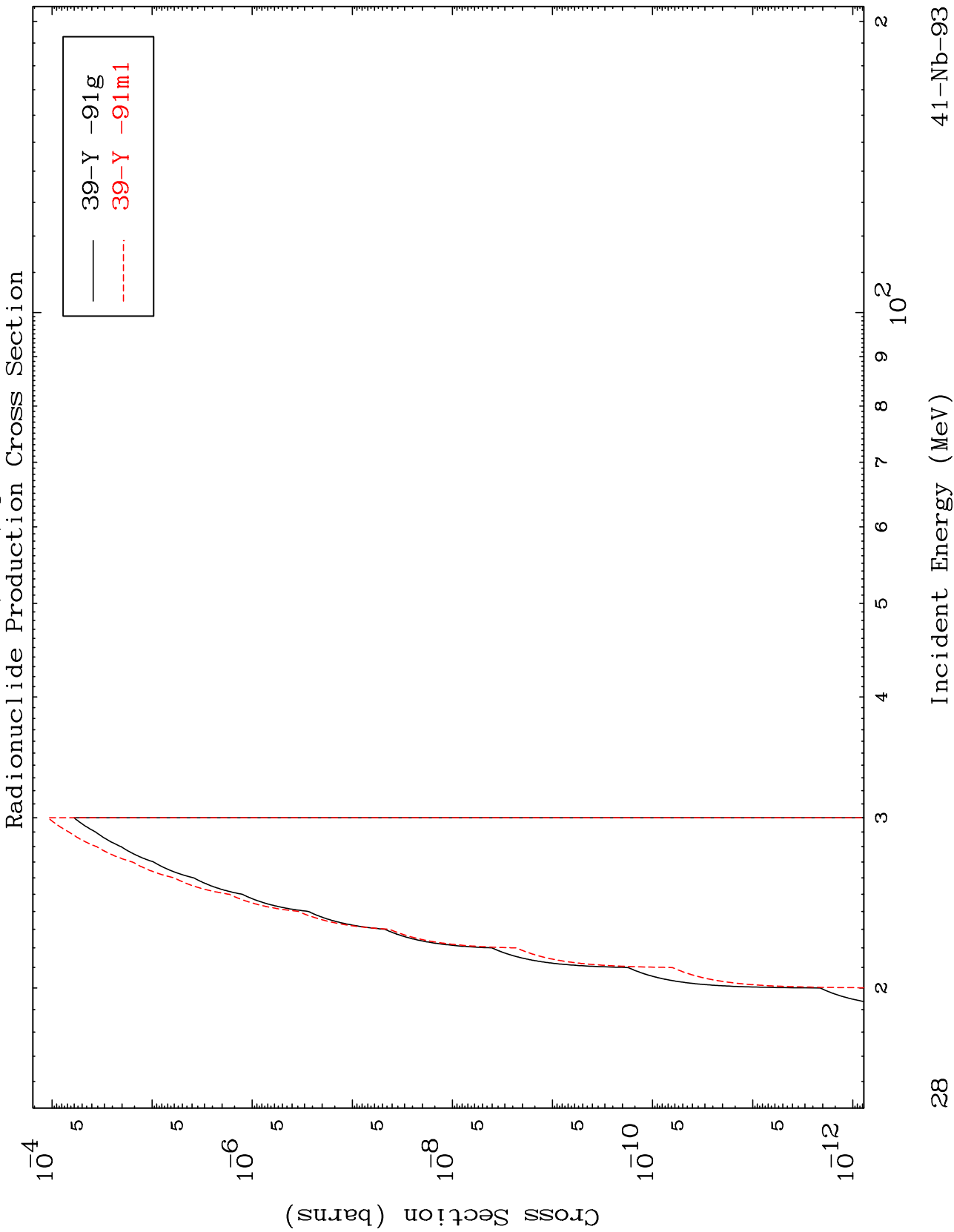
Radionuclide Production Cross Section





Radionuclide Production Cross Section

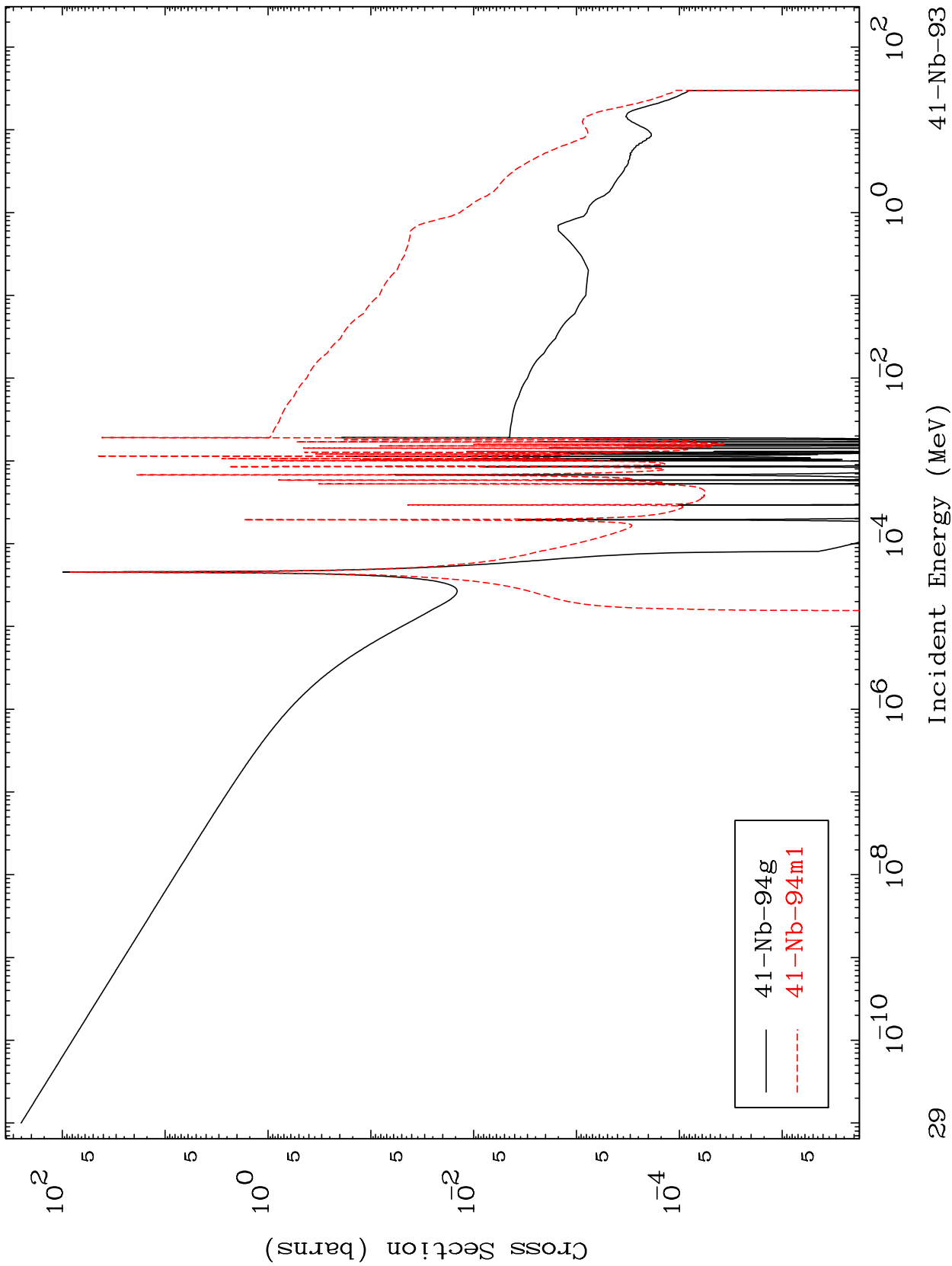




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41-Nb-93

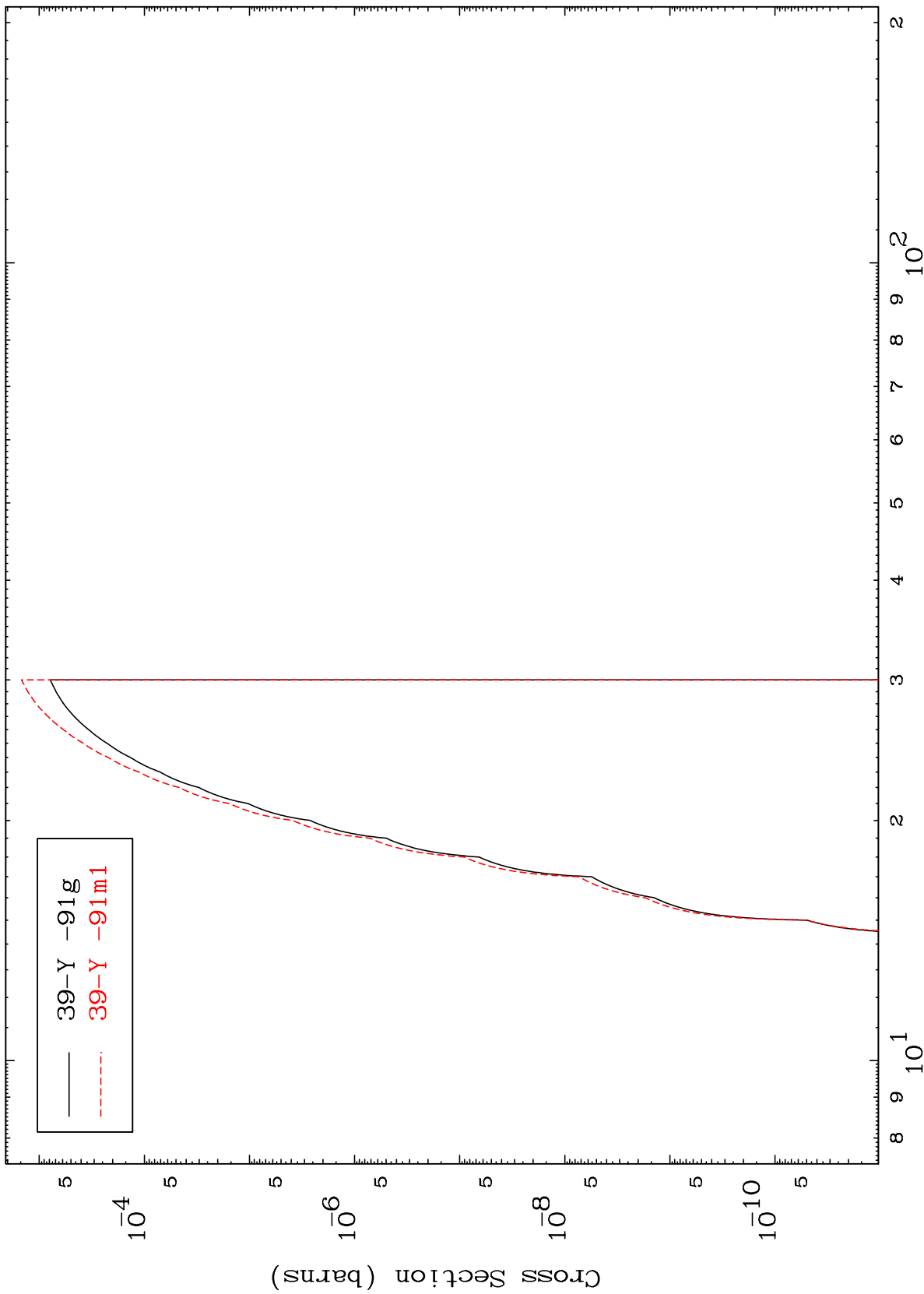
(n, γ)
Radionuclide Production Cross Section



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41-Nb-93

(n,He-3)
Radionuclide Production Cross Section



41-Nb-93

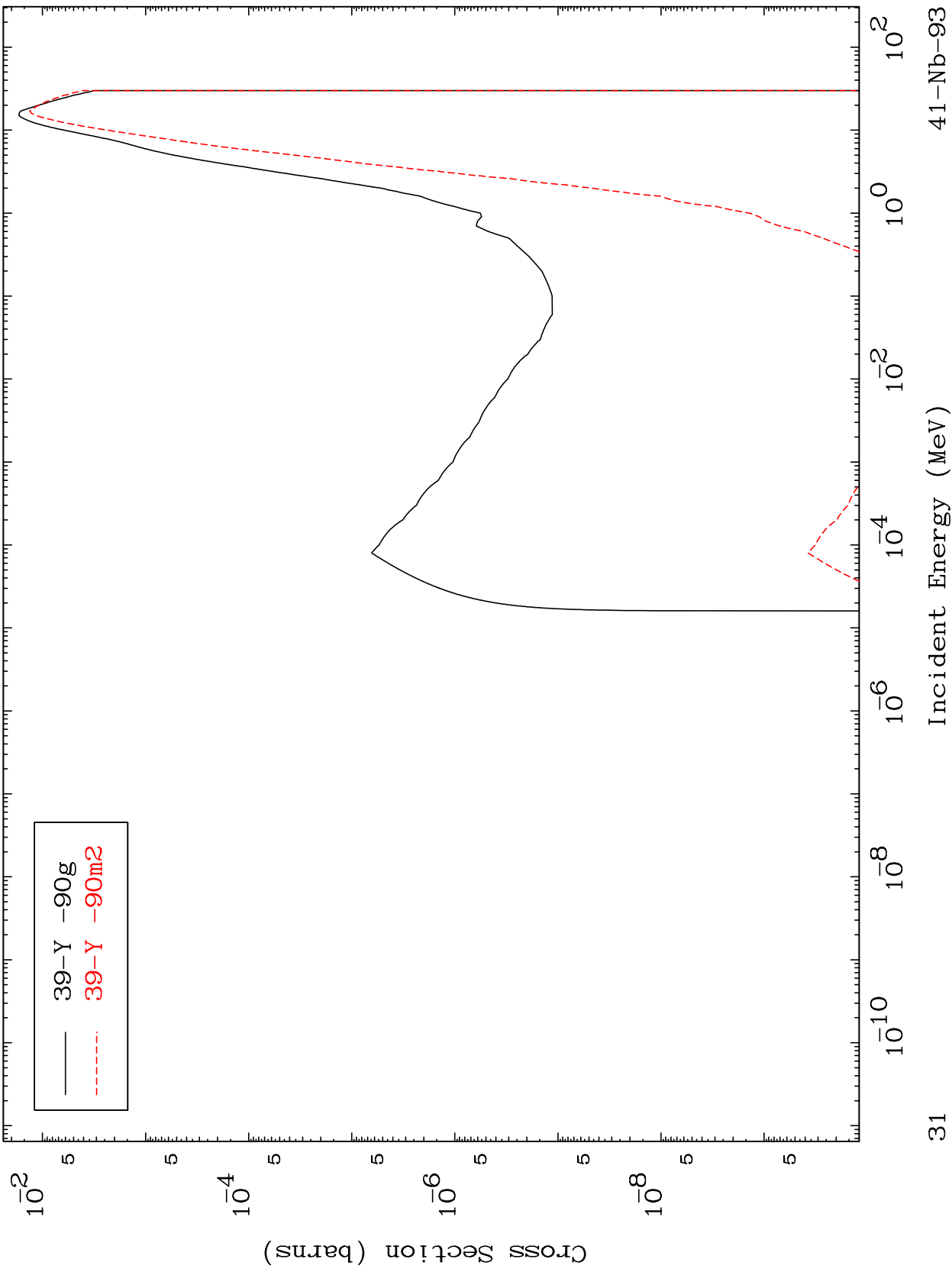
Incident Energy (MeV)

30

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41-Nb-93

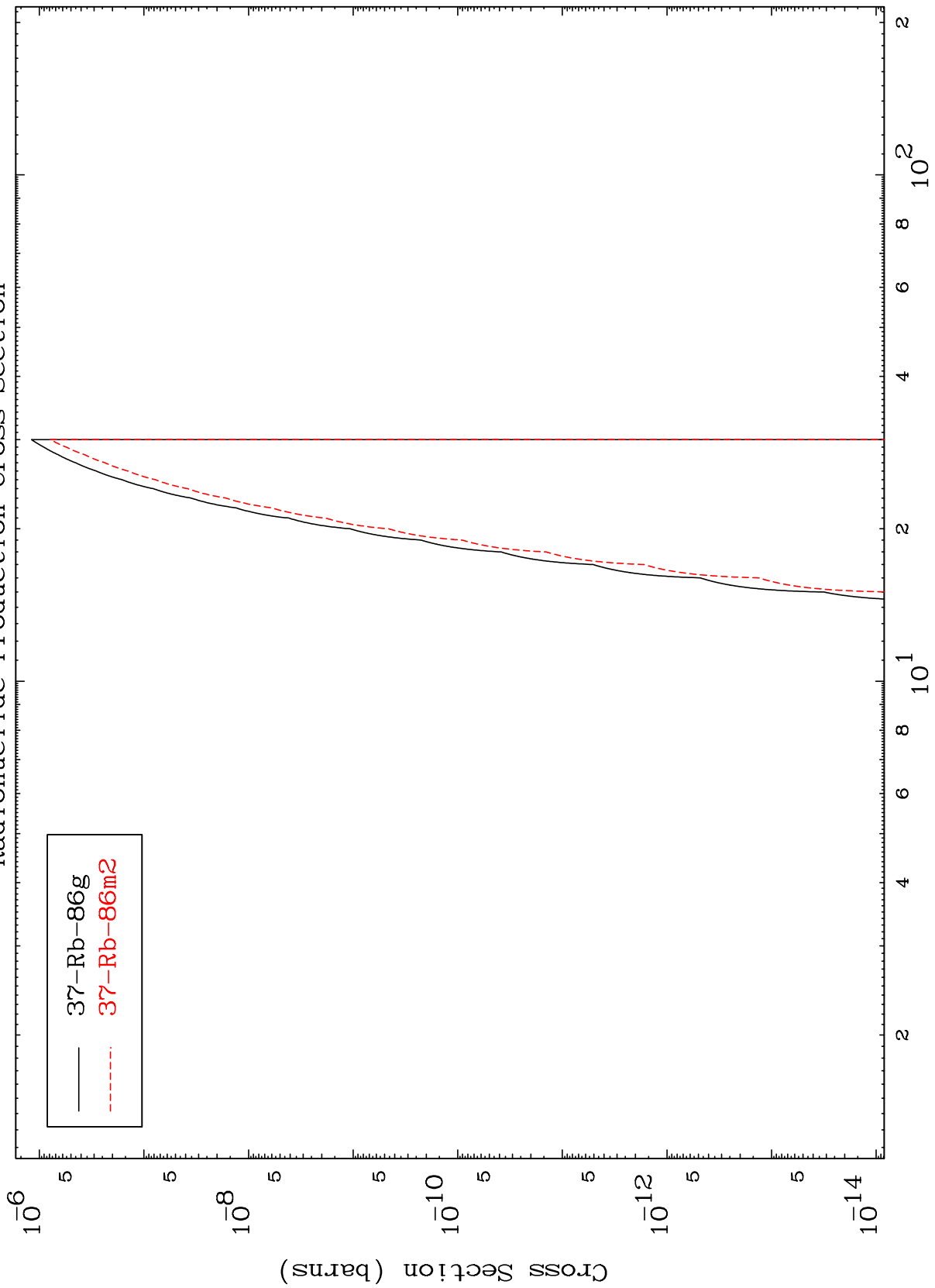
Radionuclide Production Cross Section
(n, α)



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41-Nb-93

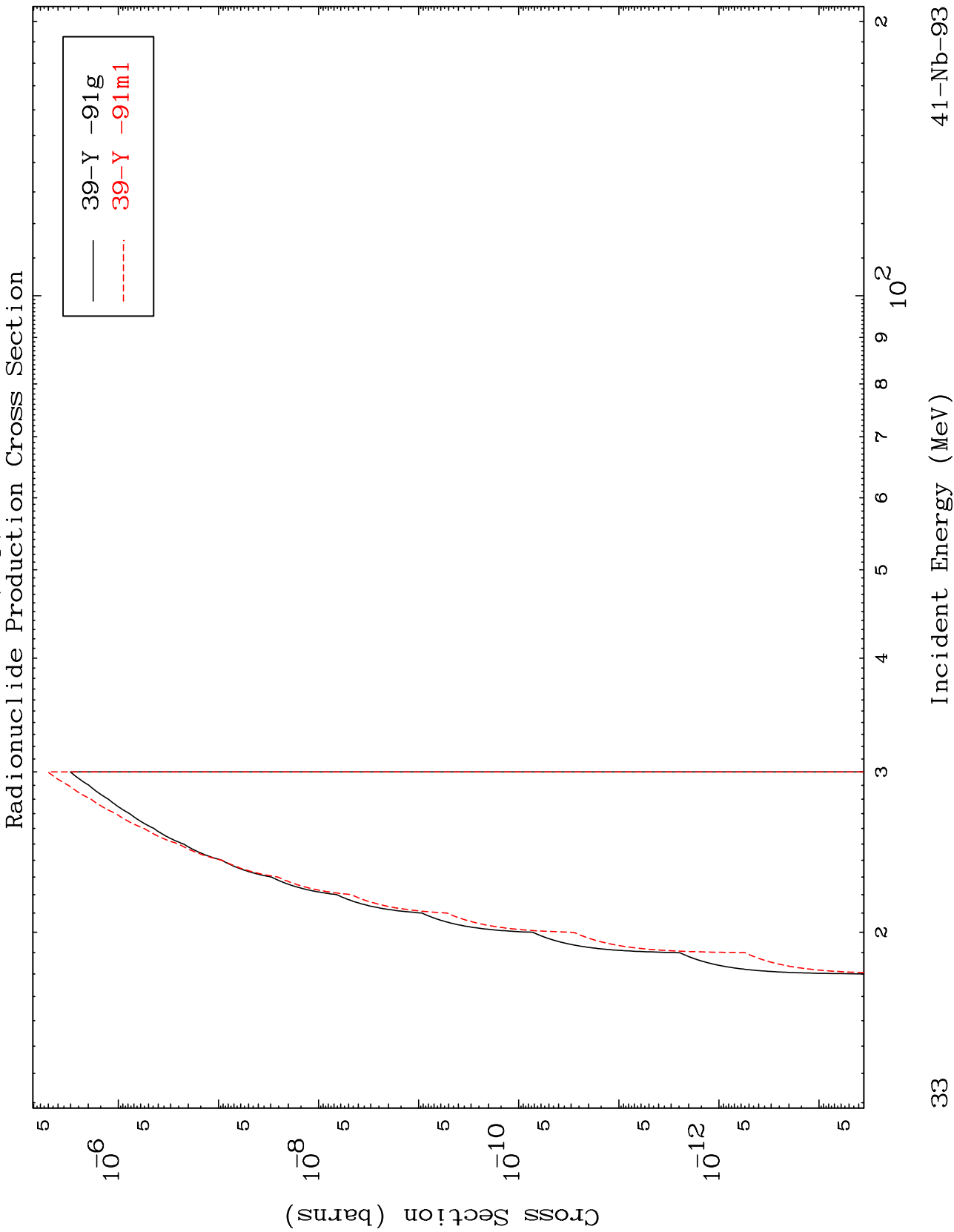
Radionuclide Production Cross Section
(n,2α)



32

41-Nb-93

Incident Energy (MeV)



Radionuclide Production Cross Section

