

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

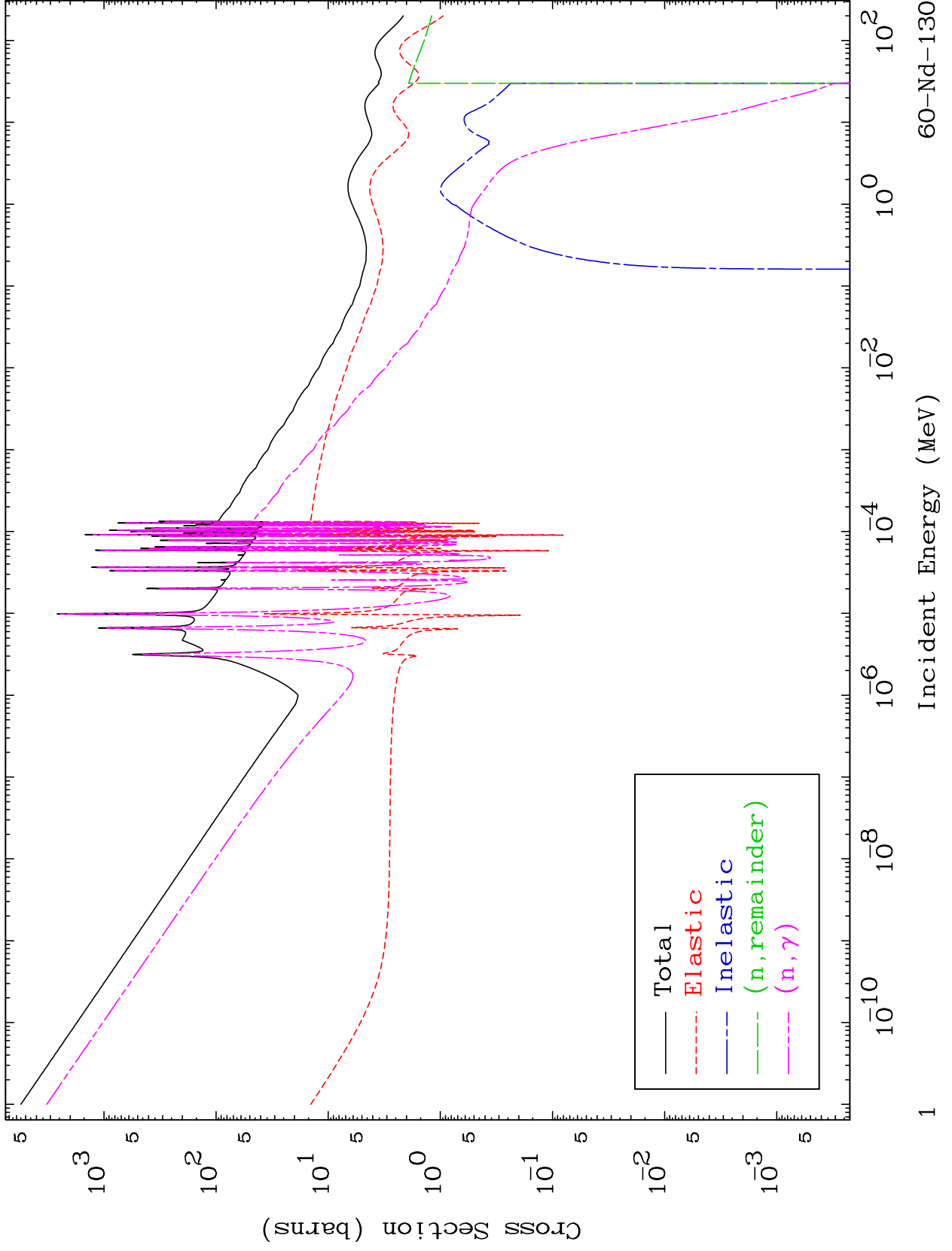
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 5989

Major  
293 Kelvin Cross Sections

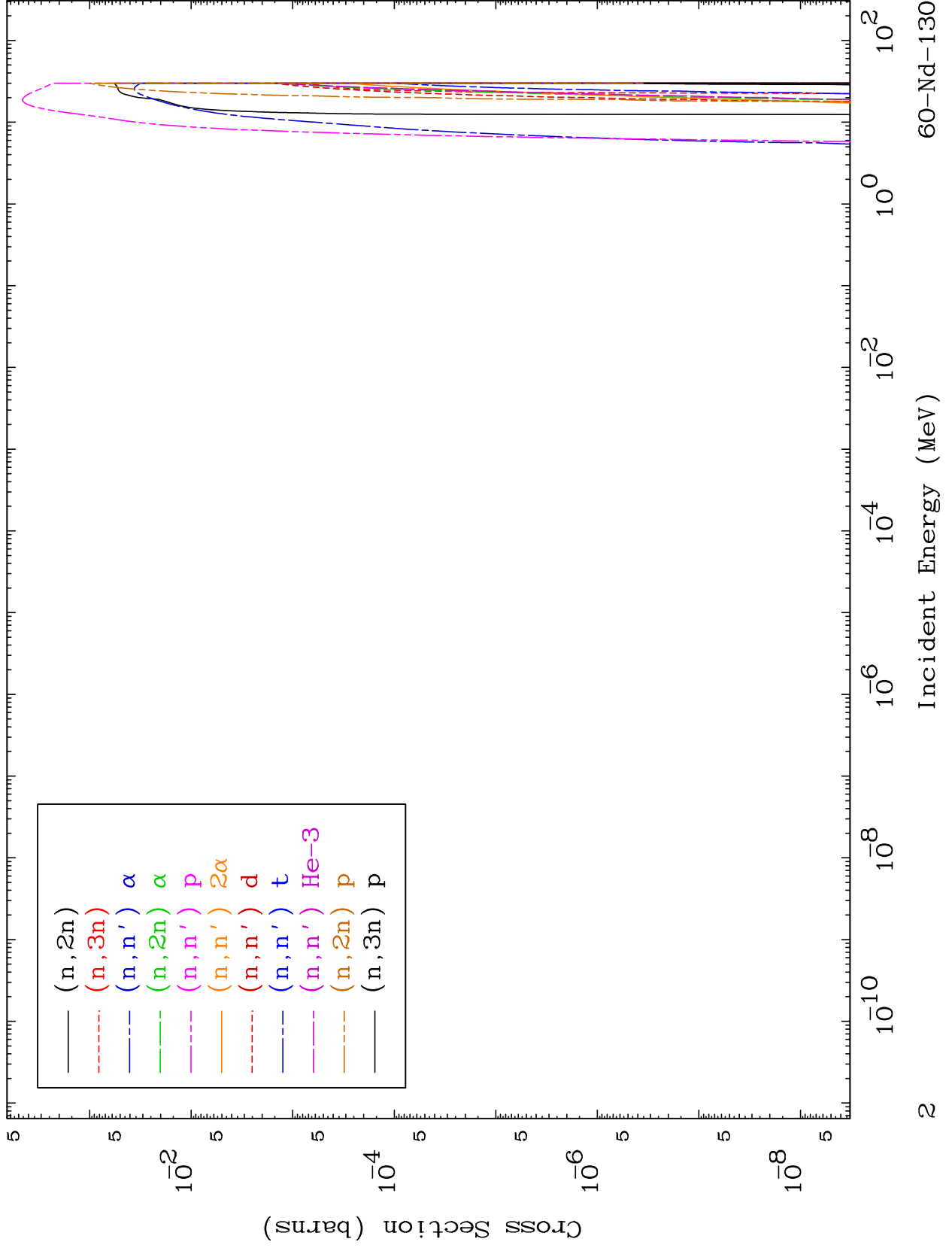
60-Nd-130



MAT 5989

Neutron Production  
293 Kelvin Cross Sections

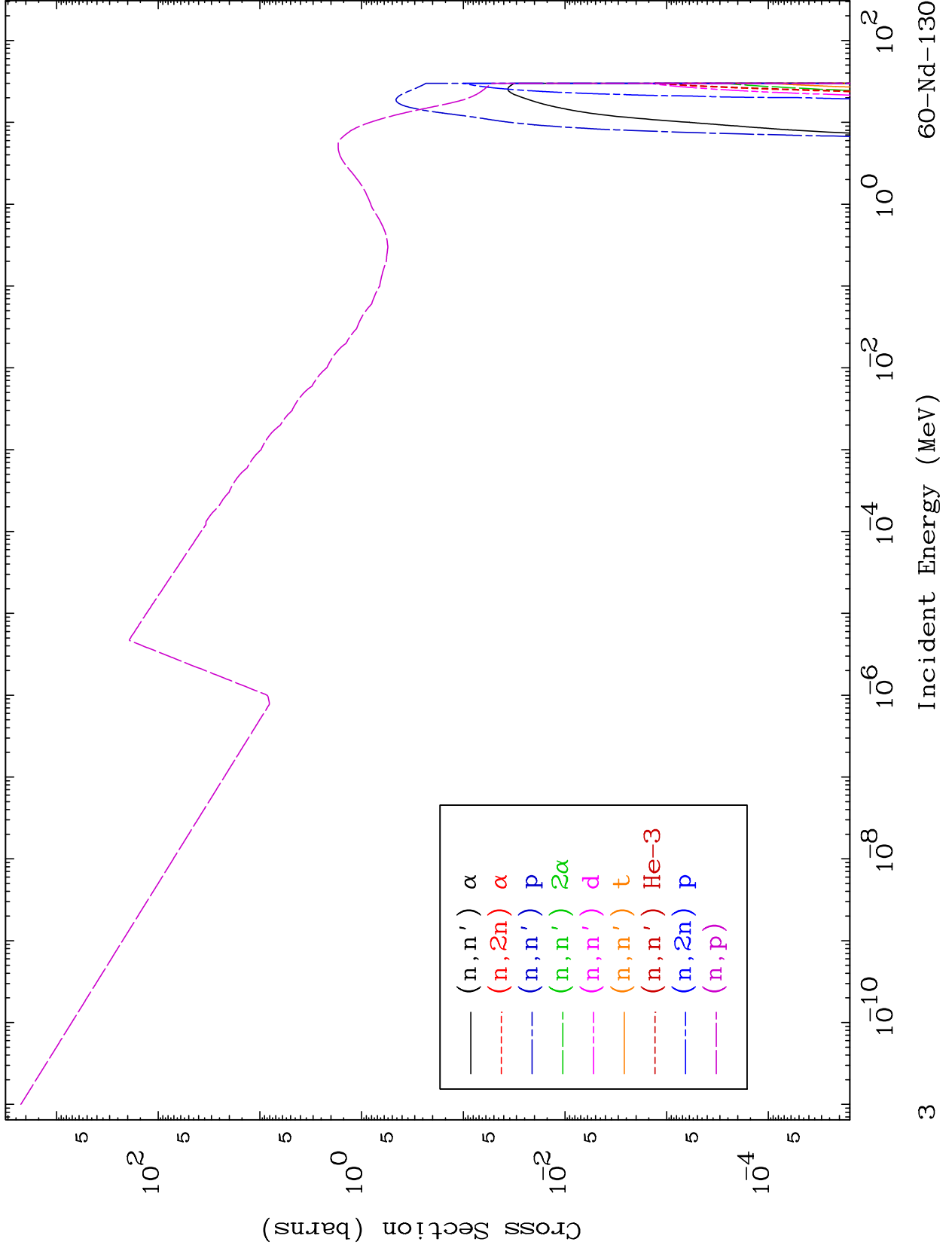
60-Nd-130



MAT 5989

Charged Particle  
293 Kelvin Cross Sections

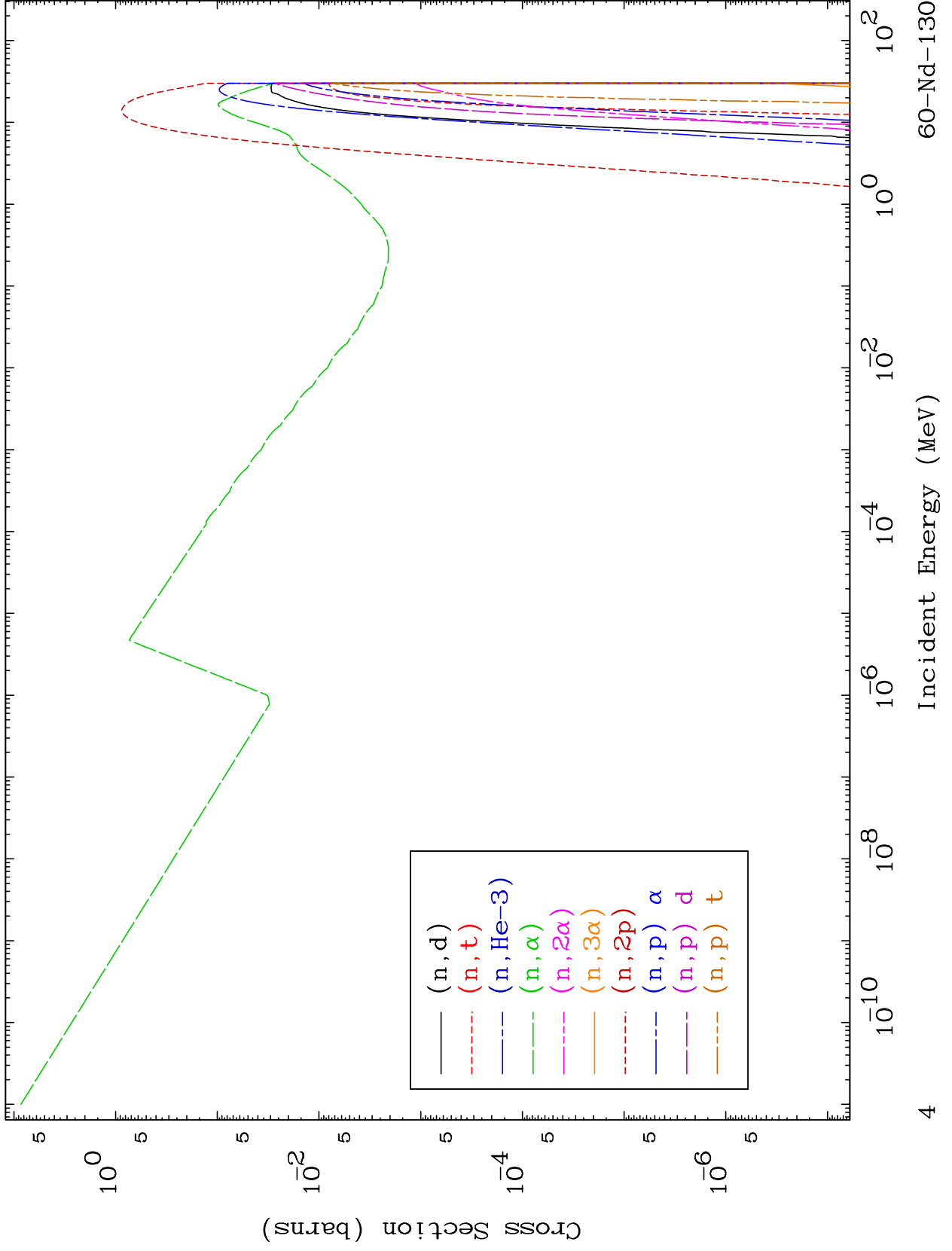
60-Nd-130



MAT 5989

Charged Particle  
293 Kelvin Cross Sections

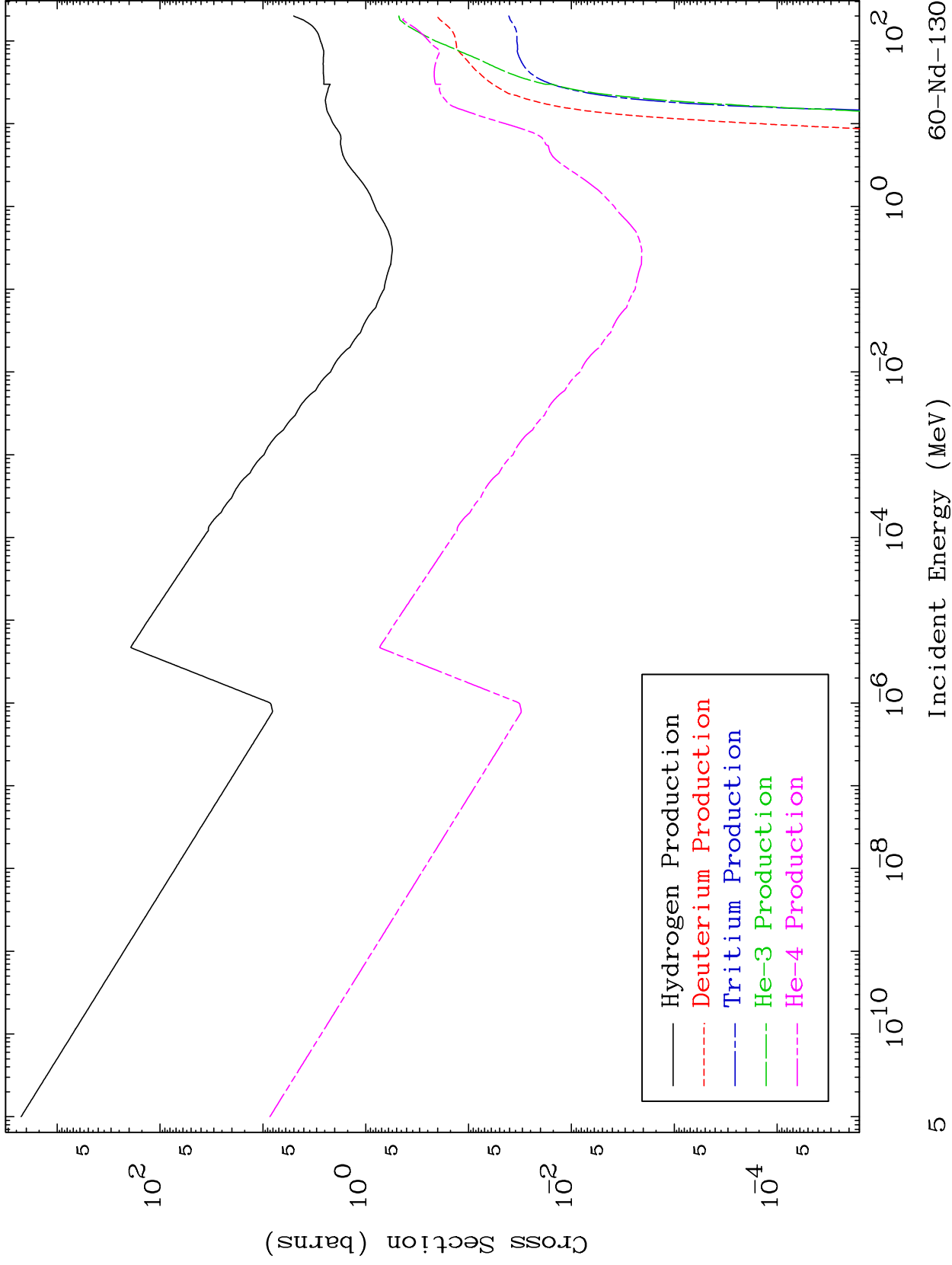
60-Nd-130

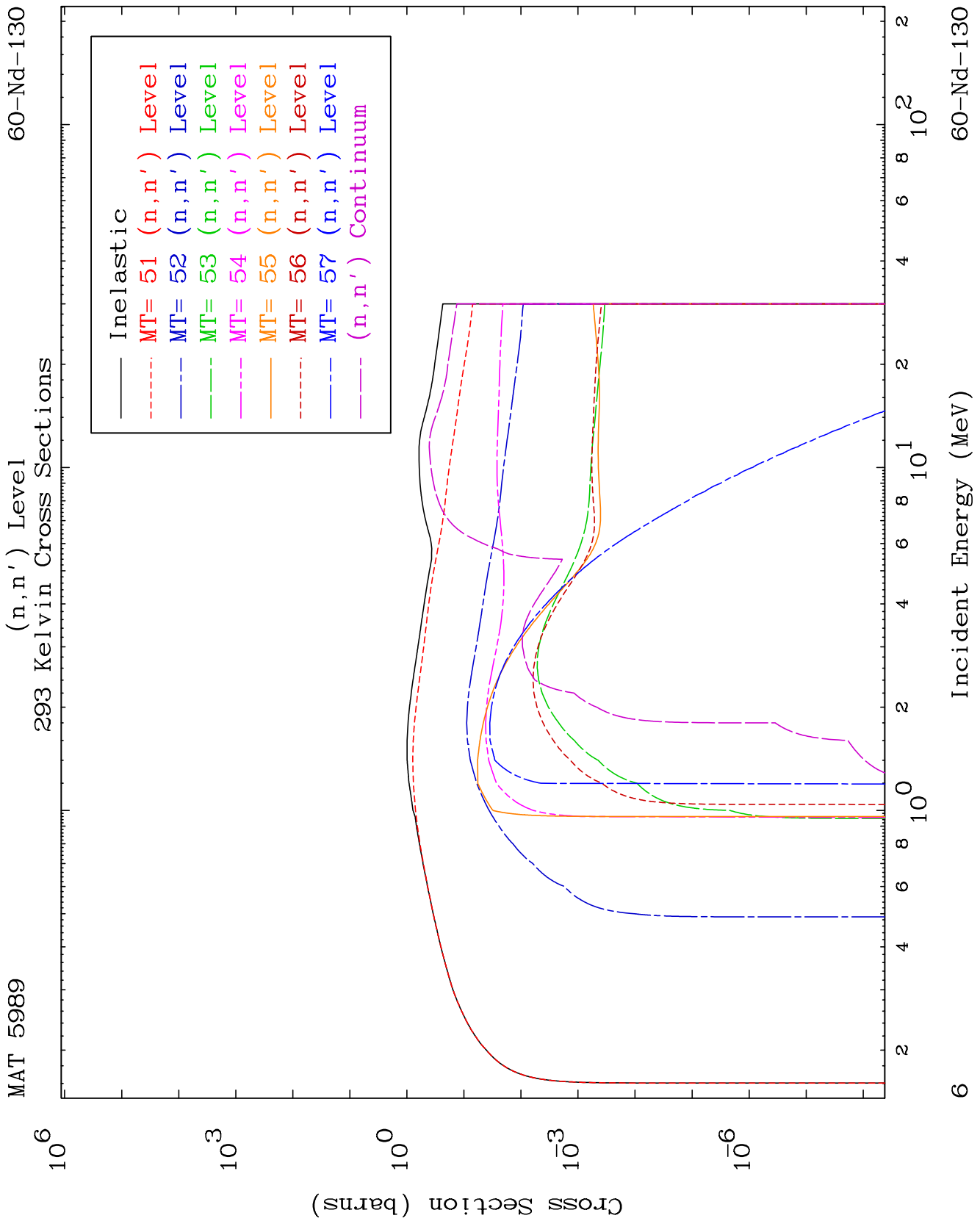


MAT 5989

Particle Production  
293 Kelvin Cross Sections

60-Nd-130

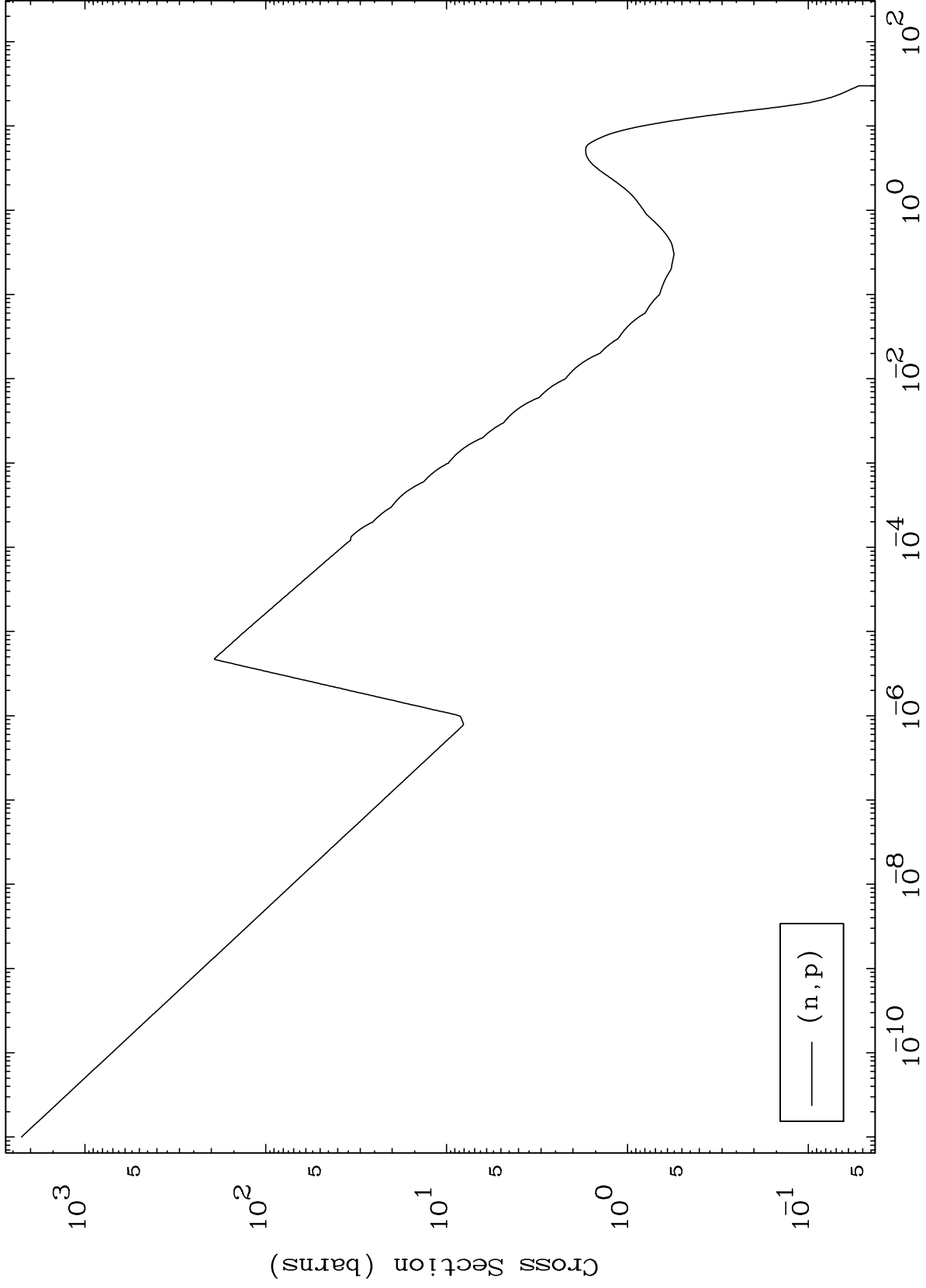




MAT 5989

(n,p) Levels  
293 Kelvin Cross Sections

60-Nd-130

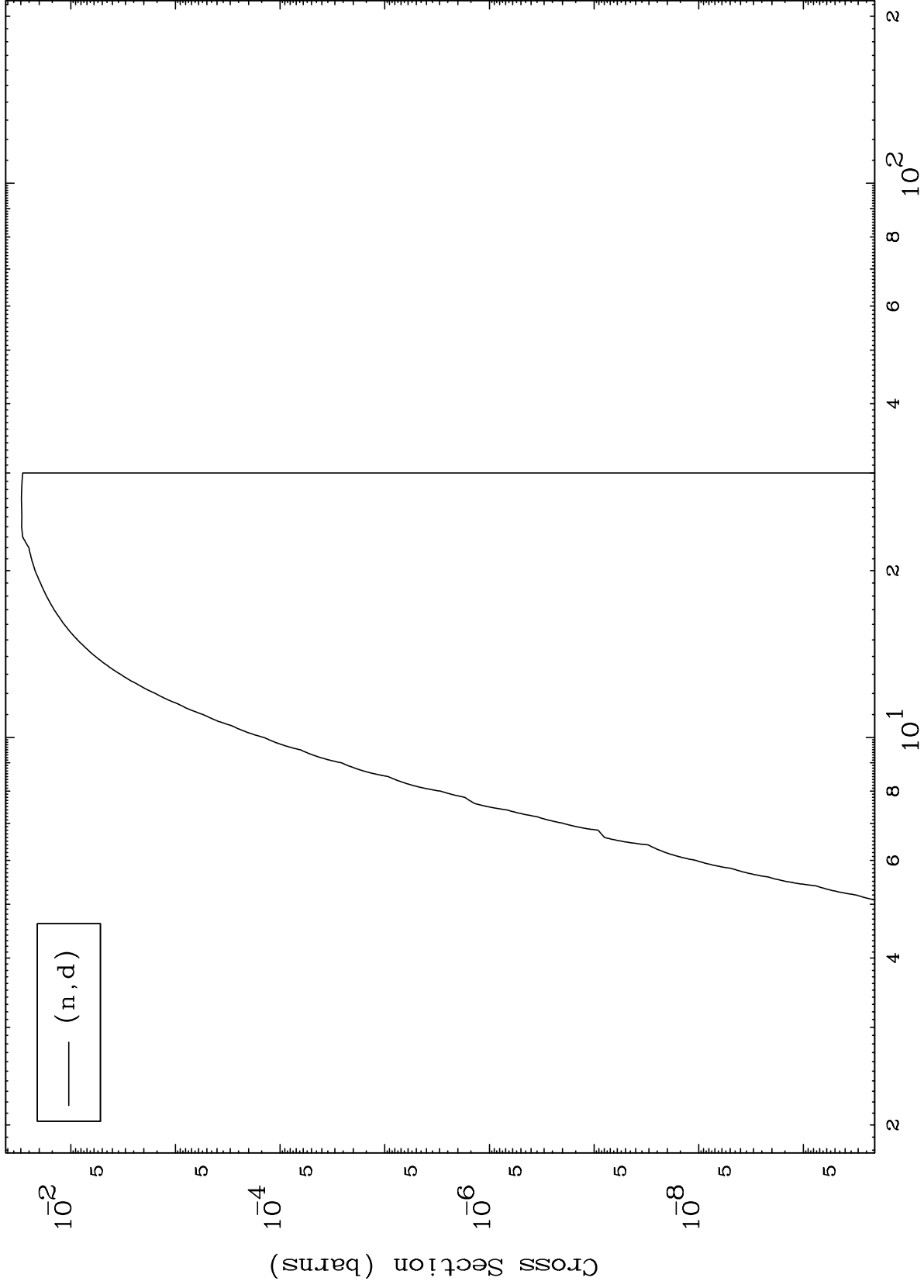




MAT 5989

(n,d) Levels  
293 Kelvin Cross Sections

60-Nd-130



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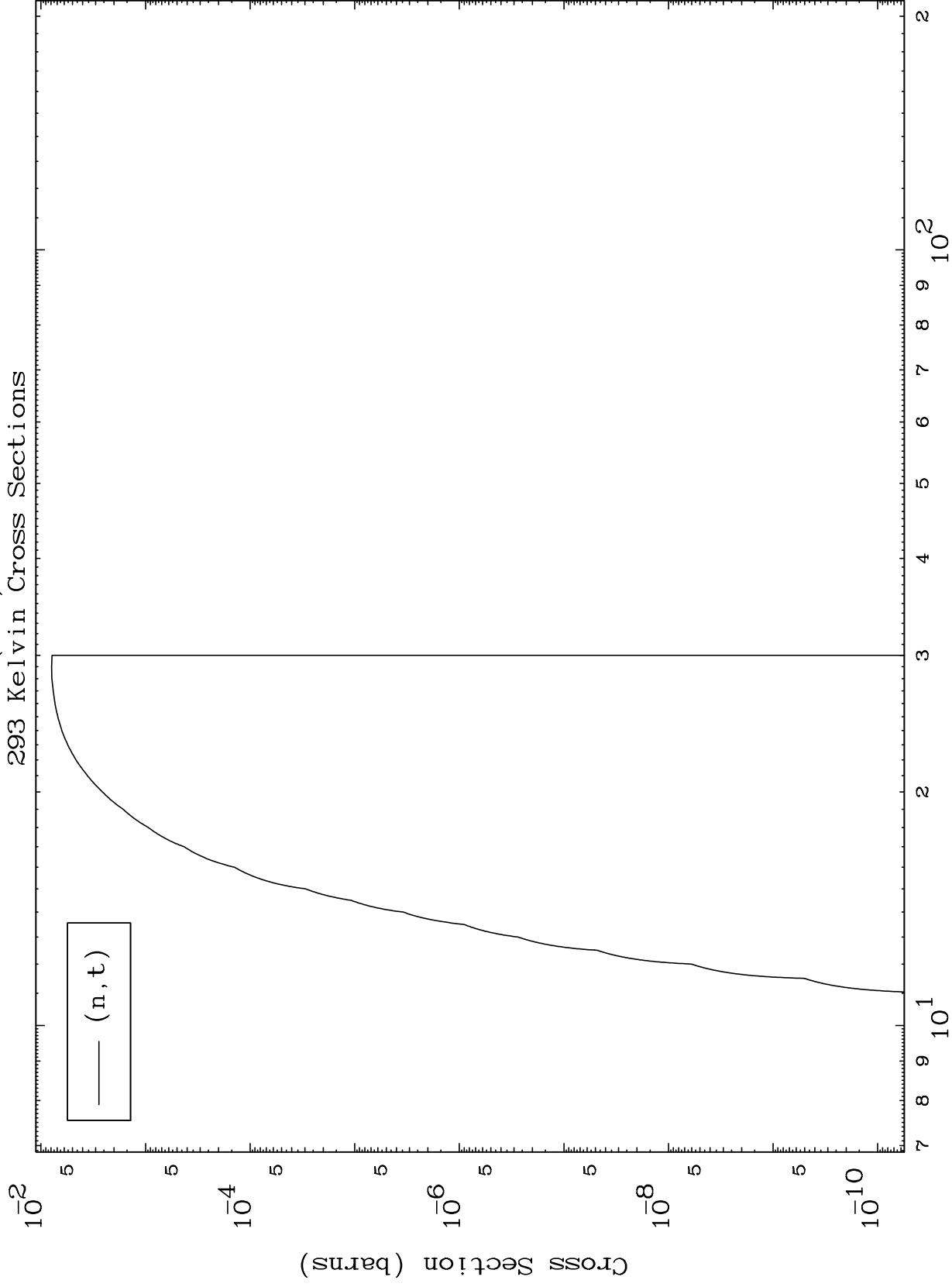
Incident Energy (MeV)

60-Nd-130

MAT 5989

(n,t) Levels  
293 Kelvin Cross Sections

60-Nd-130



9

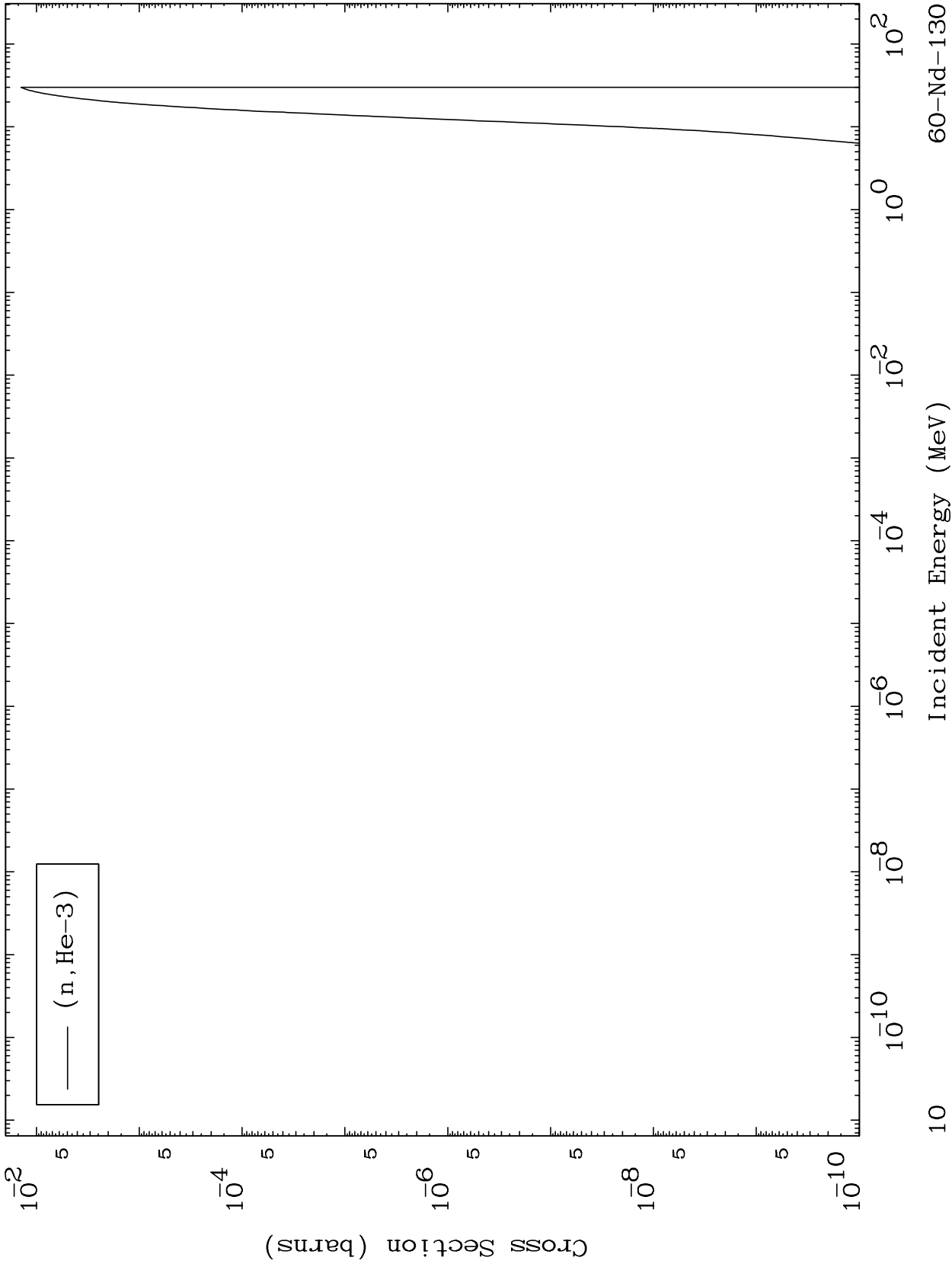
Incident Energy (MeV)

60-Nd-130

MAT 5989

(n,He3) Levels  
293 Kelvin Cross Sections

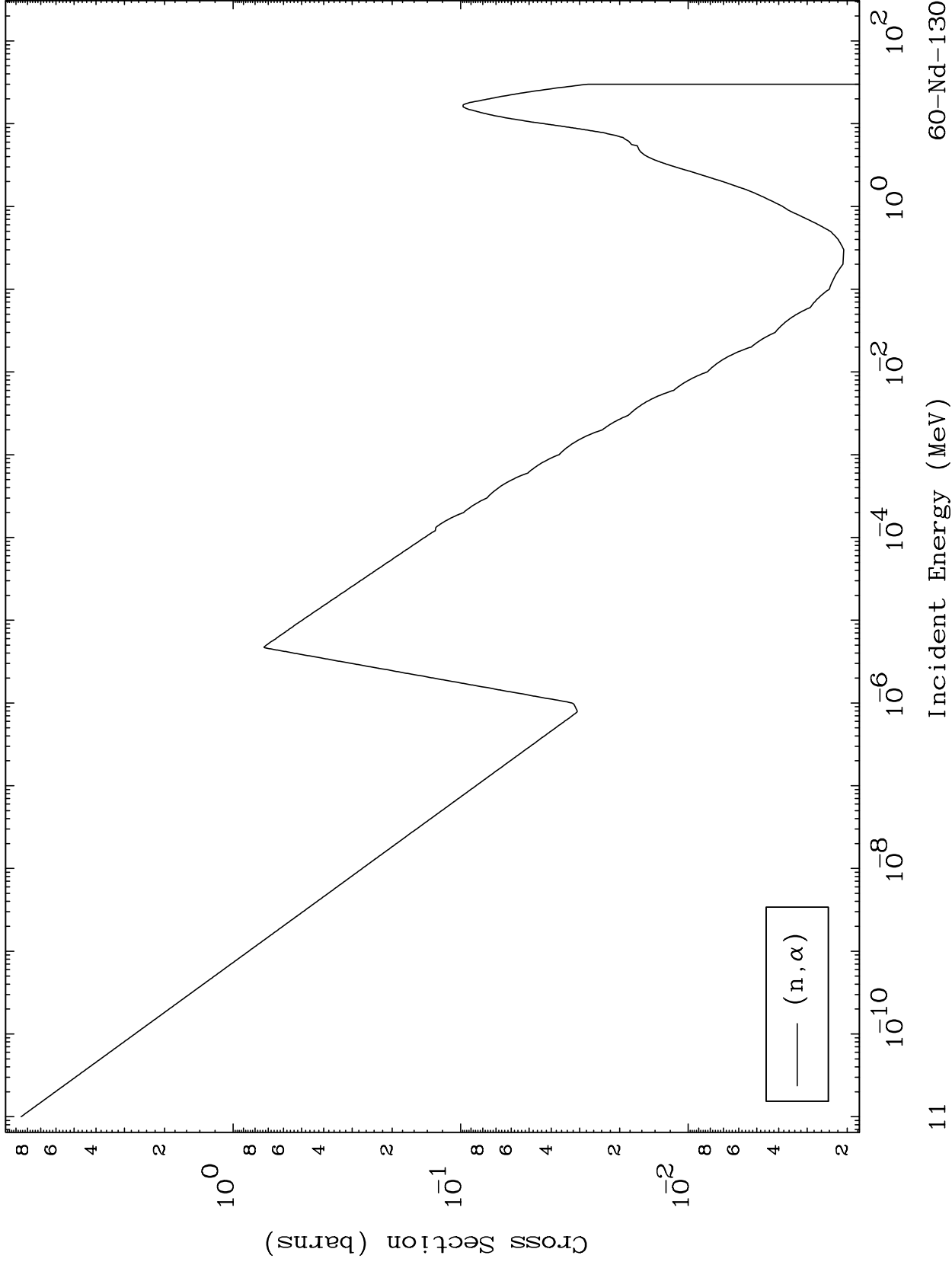
60-Nd-130

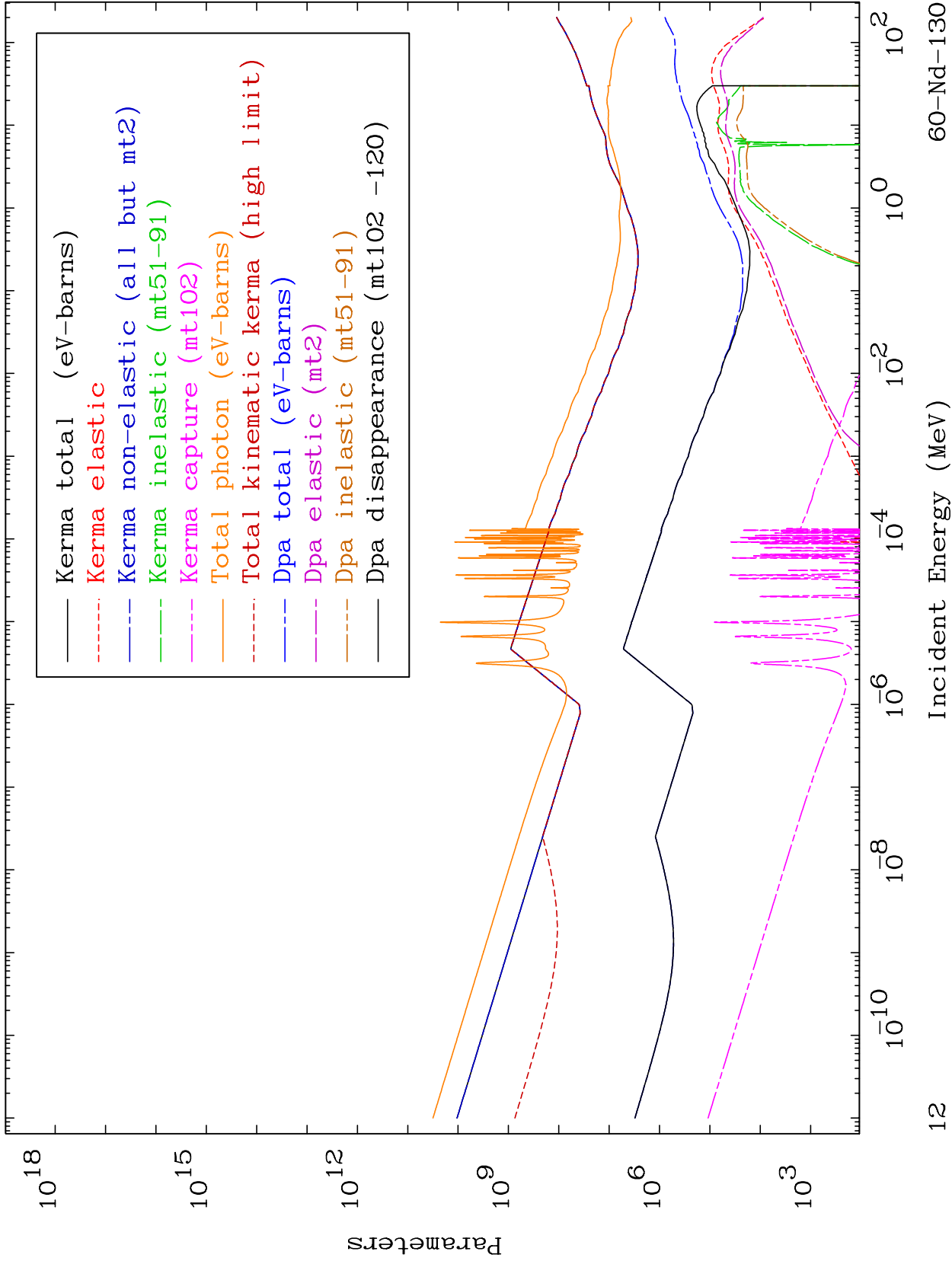


MAT 5989

(n,α) Levels  
293 Kelvin Cross Sections

60-Nd-130

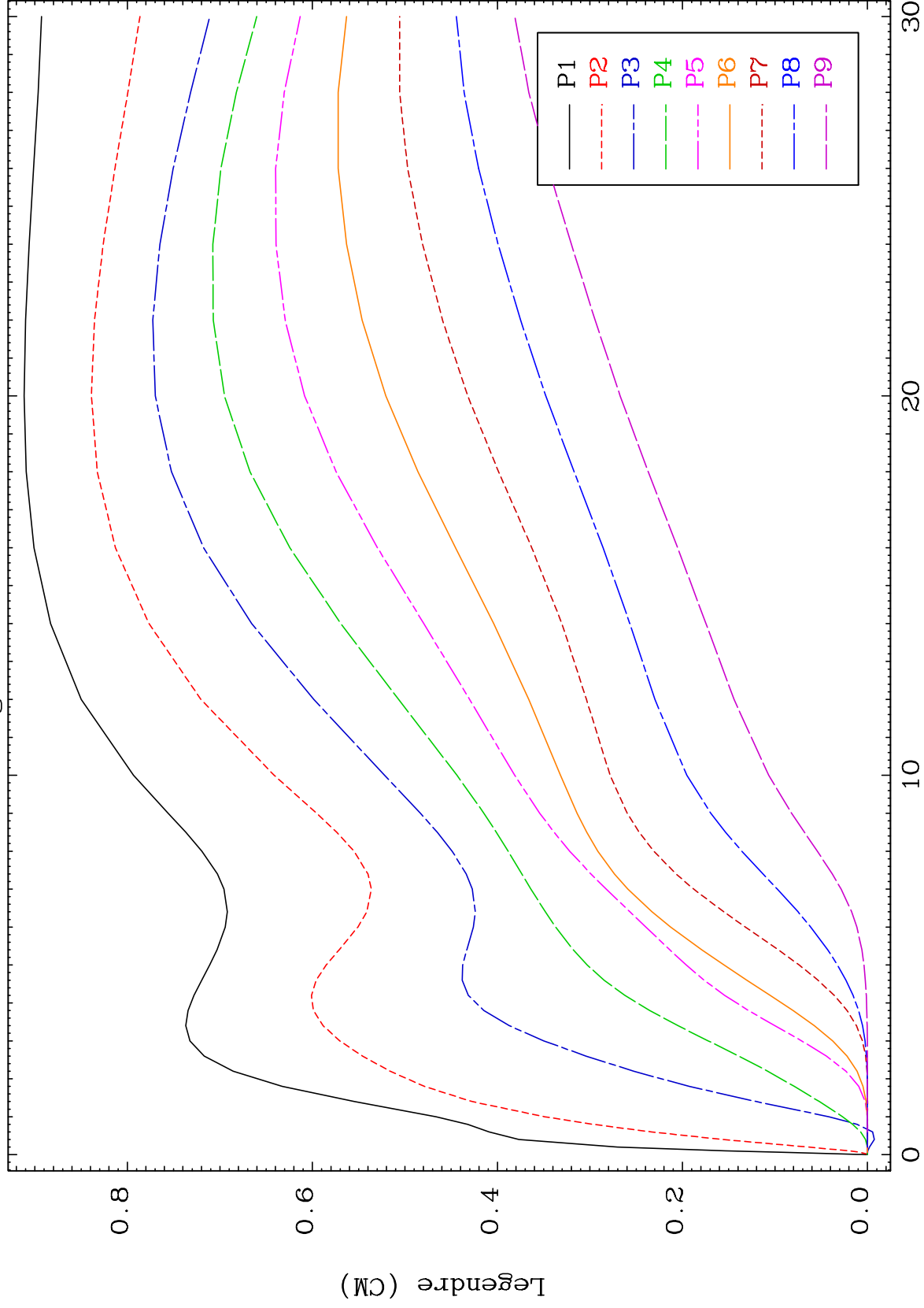




MAT 5989

Elastic Legendre Coefficients

60-Nd-130



13

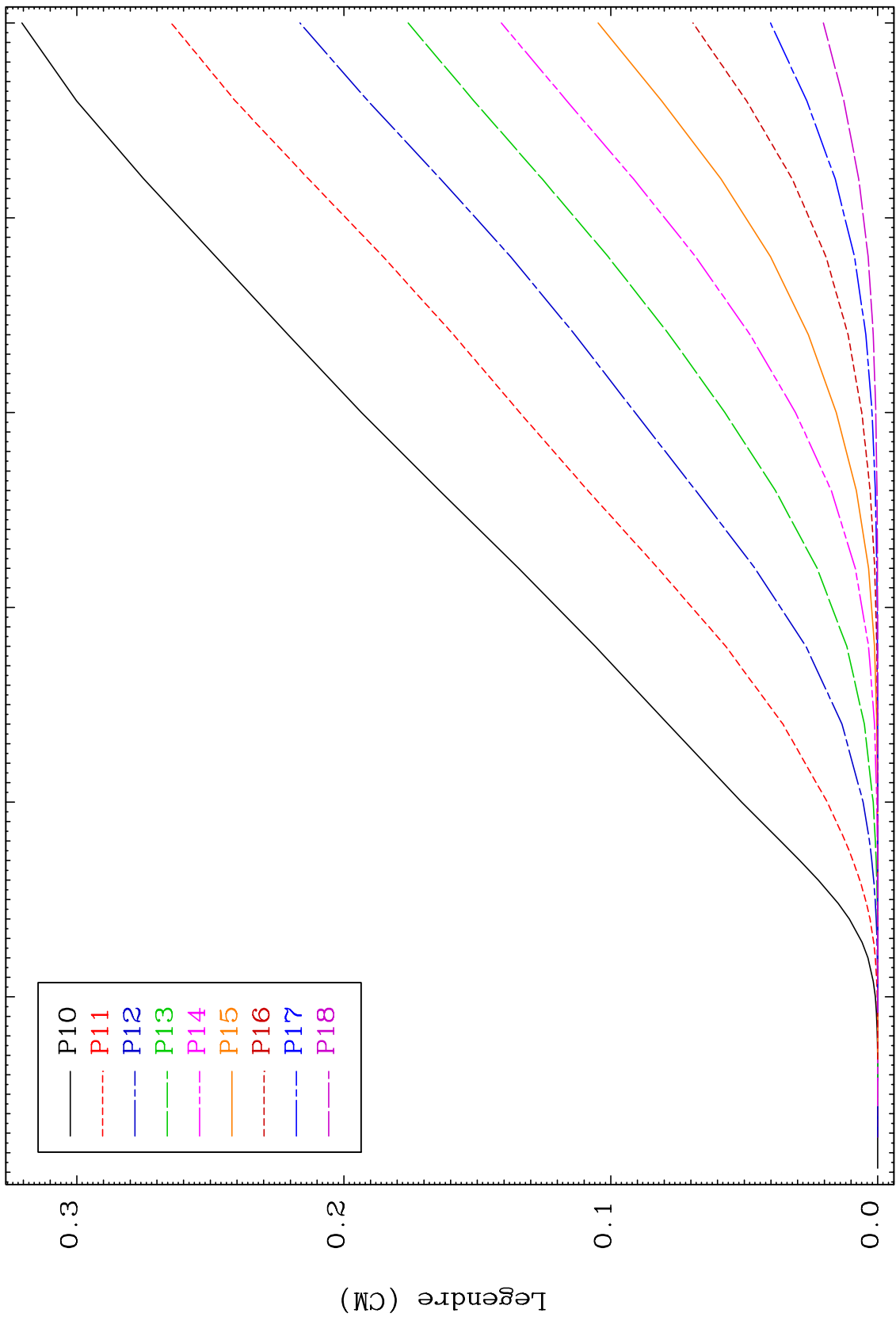
Incident Energy (MeV)

60-Nd-130

MAT 5989

Elastic Legendre Coefficients

60-Nd-130



14

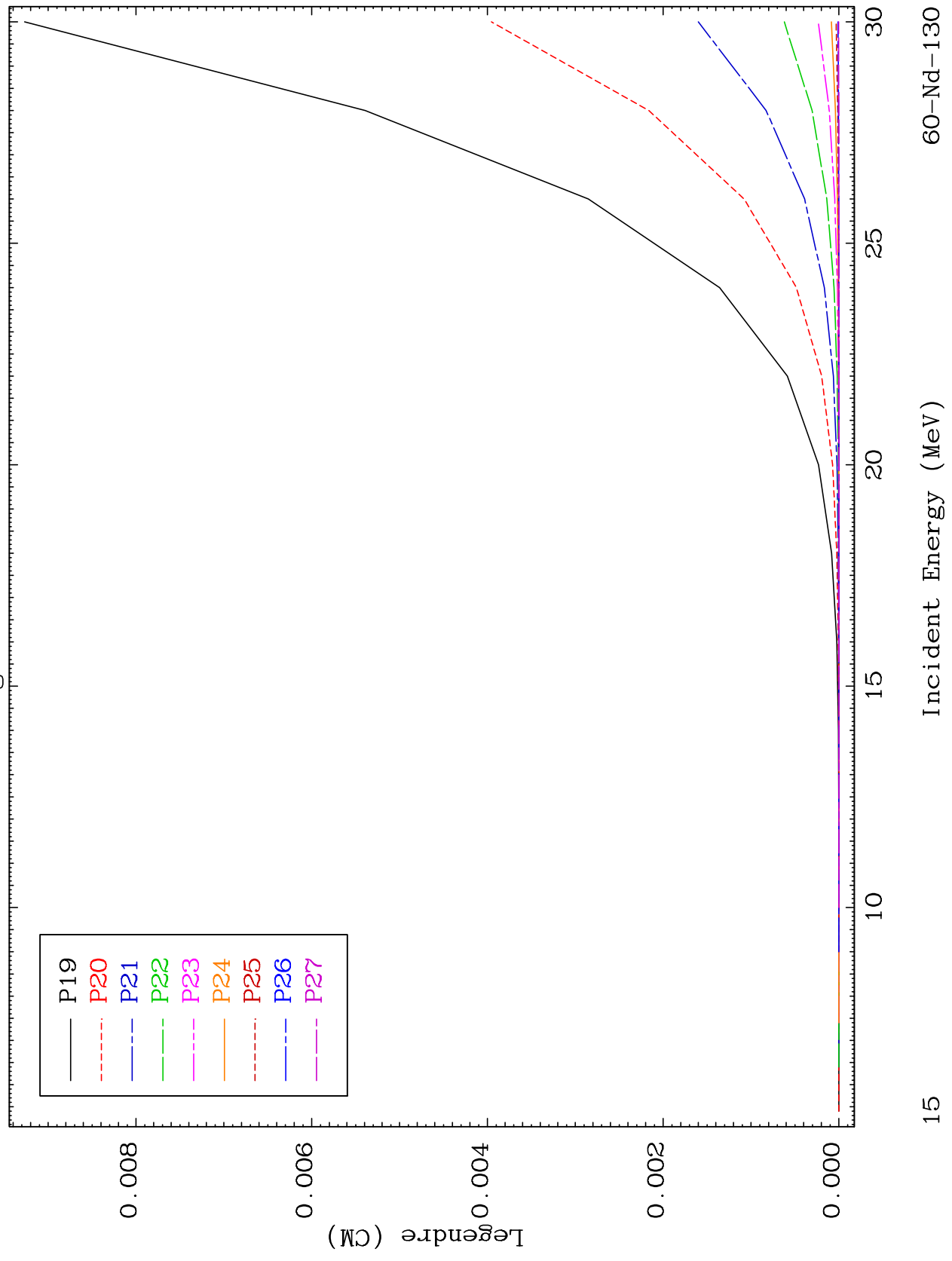
Incident Energy (MeV)

60-Nd-130

MAT 5989

### Elastic Legendre Coefficients

60-Nd-130



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Incident Energy (MeV)

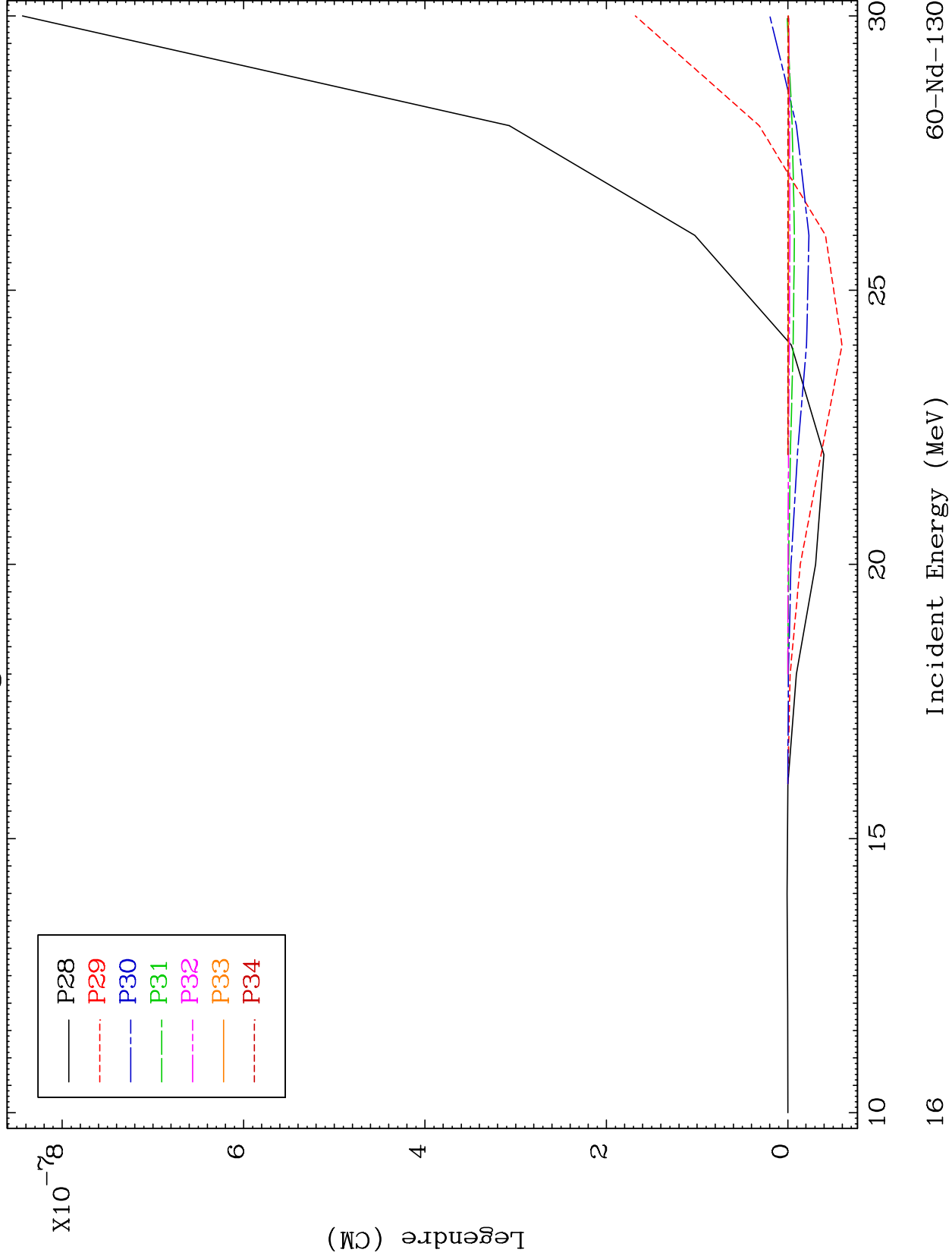
60-Nd-130



MAT 5989

Elastic Legendre Coefficients

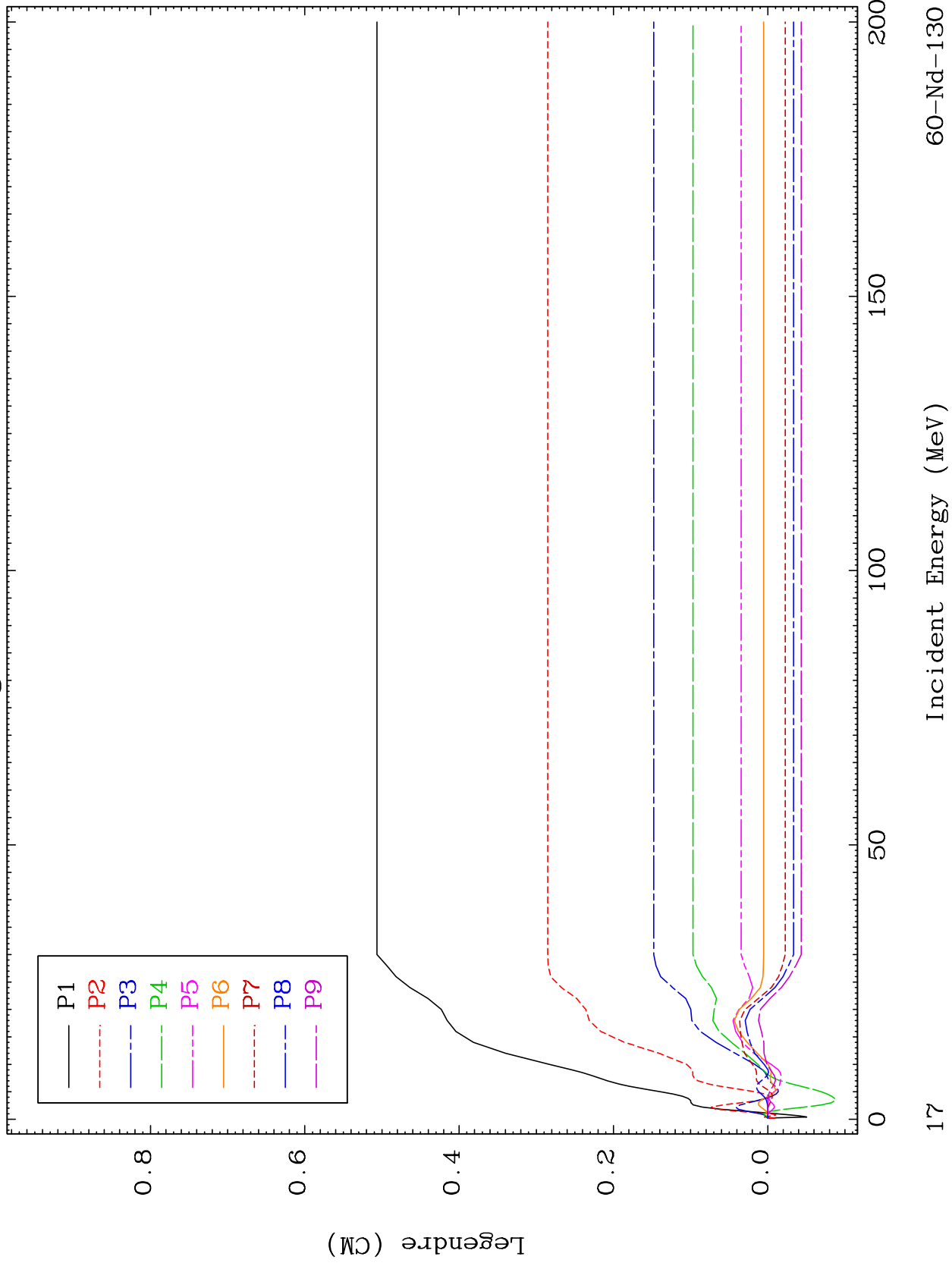
60-Nd-130



MAT 5989

MT= 51 (n,n') Level  
Legendre Coefficients

60-Nd-130



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Incident Energy (MeV)

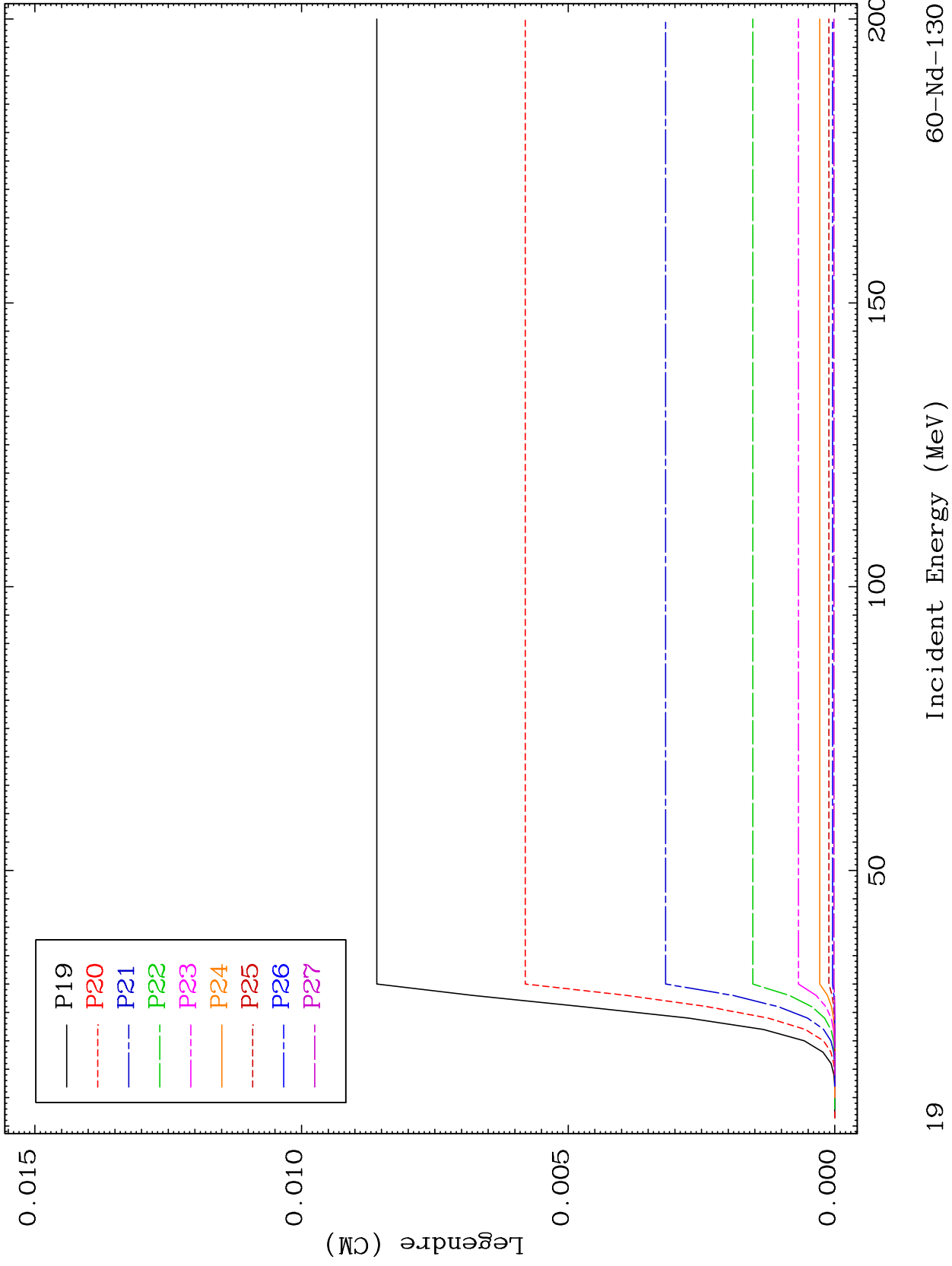
60-Nd-130



MAT 5989

MT= 51 (n,n') Level  
Legendre Coefficients

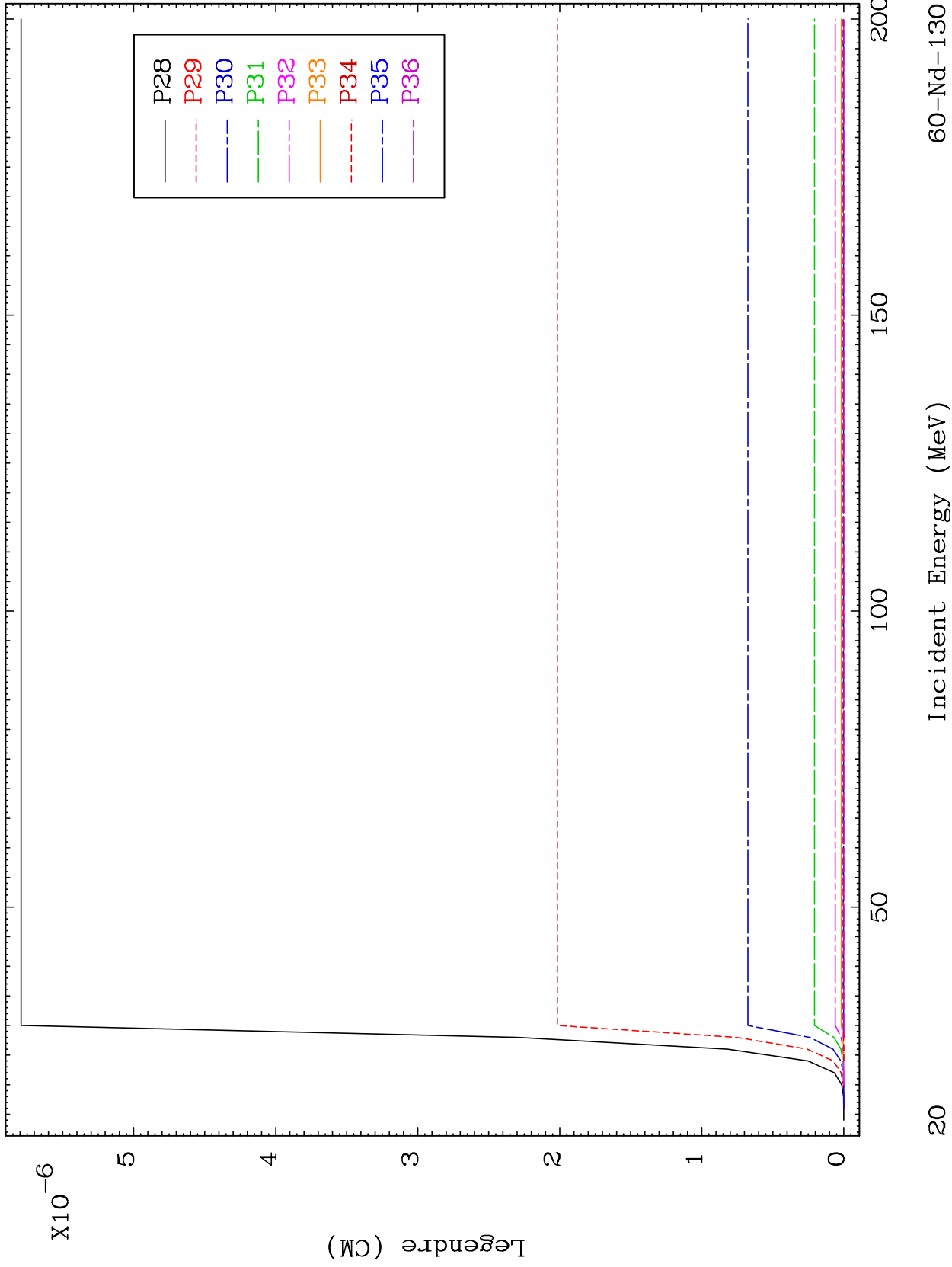
60-Nd-130



MAT 5989

MT= 51 (n,n') Level  
Legendre Coefficients

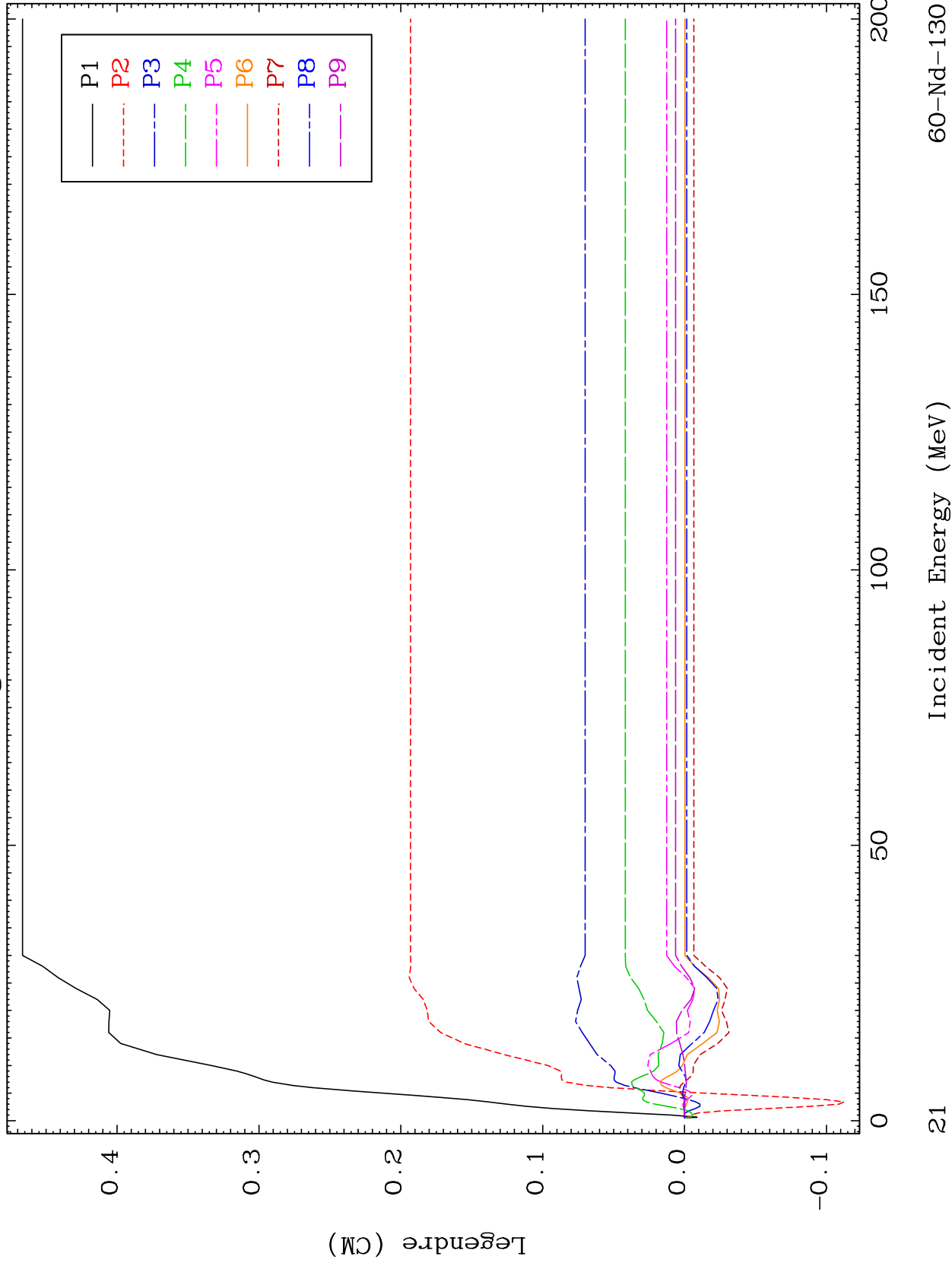
60-Nd-130



MAT 5989

MT= 52 (n,n') Level  
Legendre Coefficients

60-Nd-130

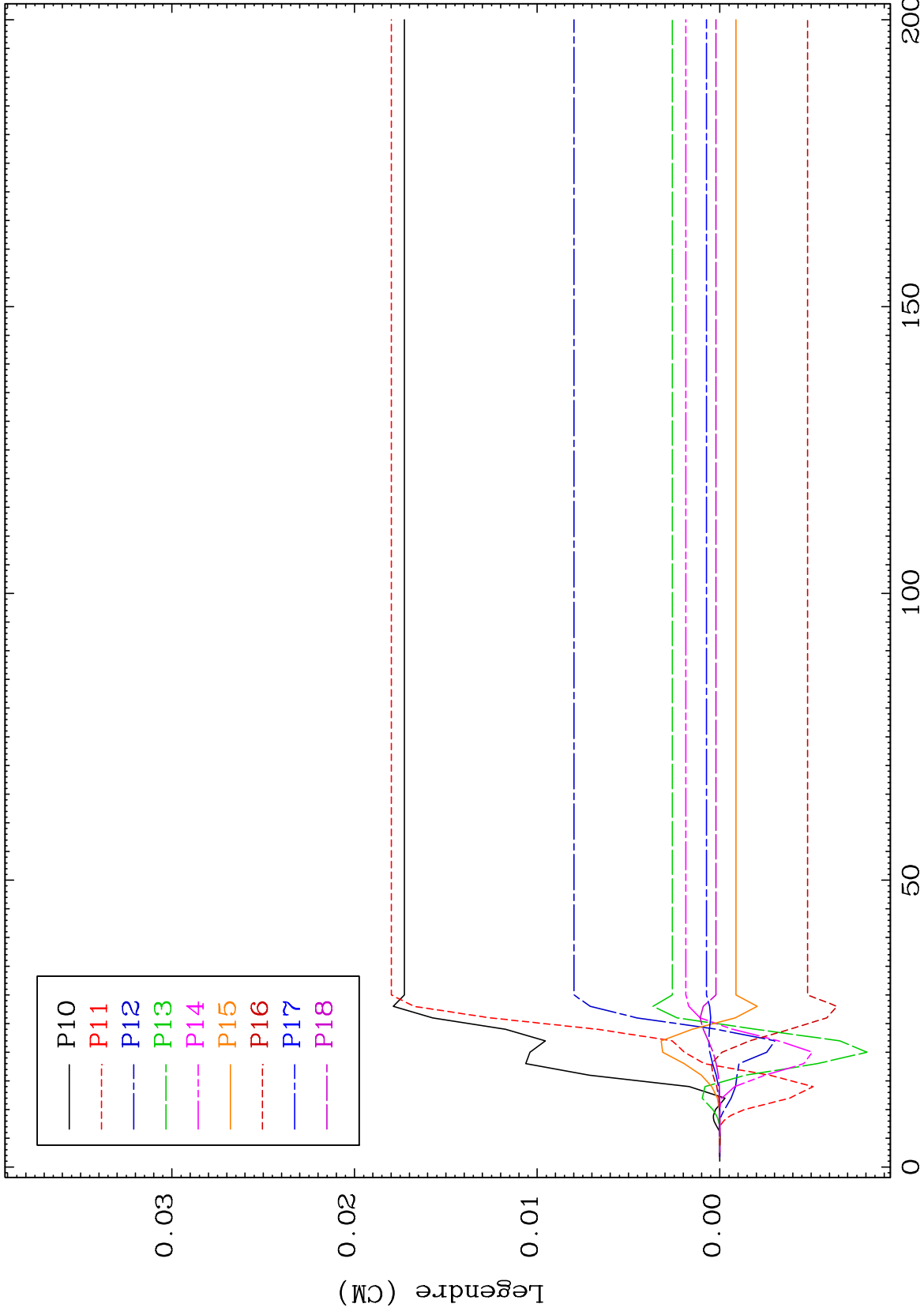


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MAT 5989

MT= 52 (n,n') Level  
Legendre Coefficients

60-Nd-130



22

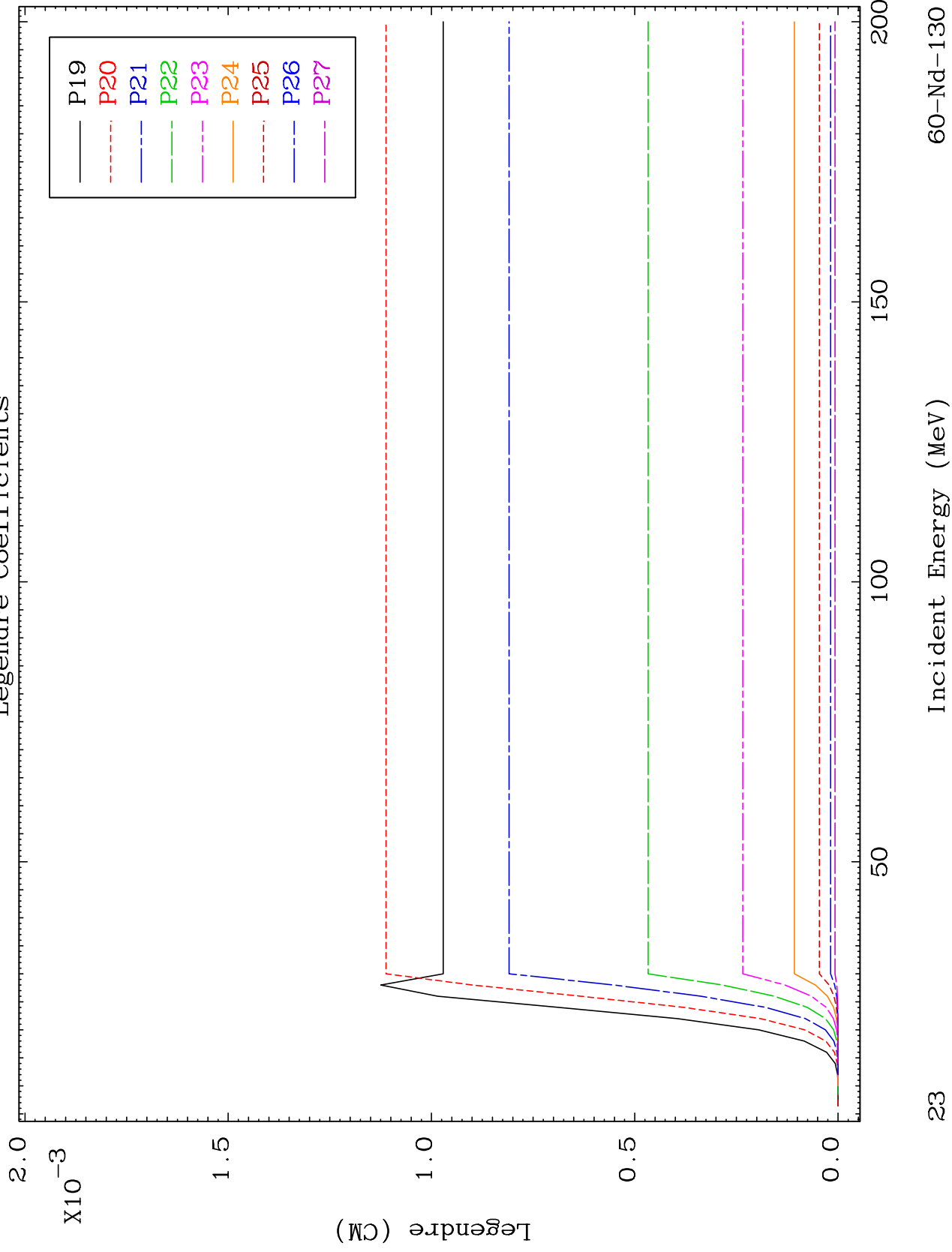
Incident Energy (MeV)

60-Nd-130

MAT 5989

MT= 52 (n,n') Level  
Legendre Coefficients

60-Nd-130



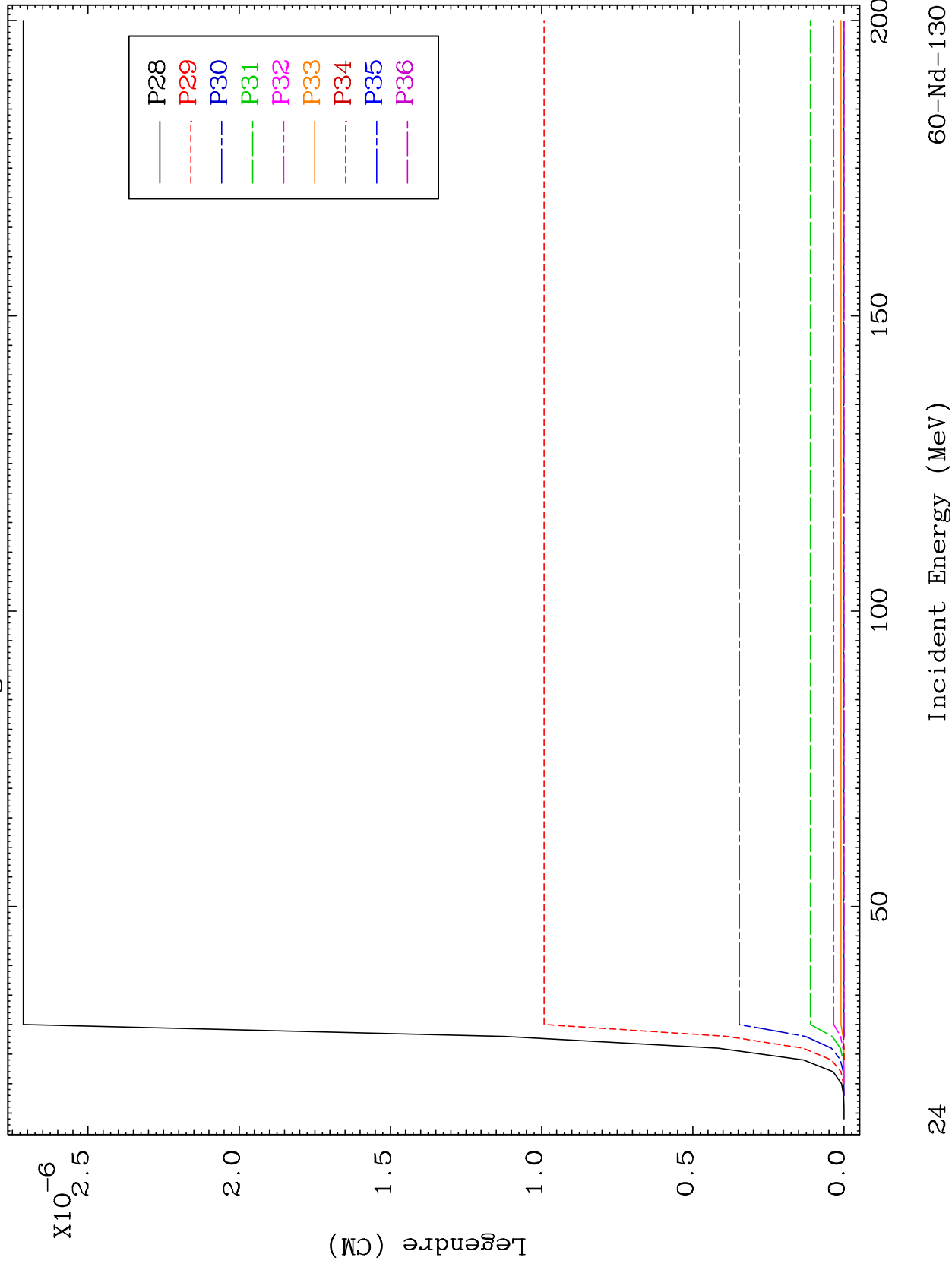
23



MAT 5989

MT= 52 (n,n') Level  
Legendre Coefficients

60-Nd-130



24

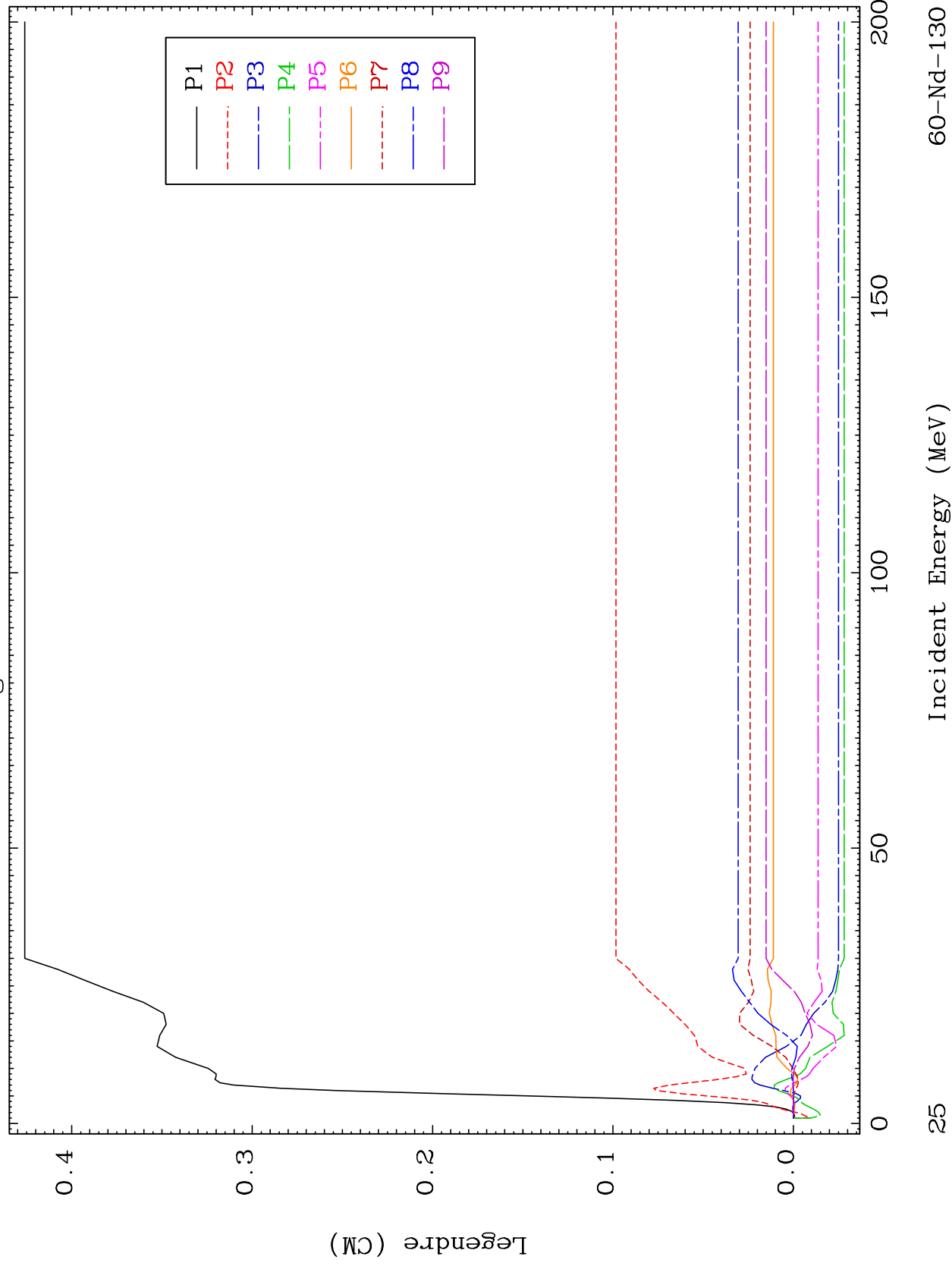
Incident Energy (MeV)

60-Nd-130

MAT 5989

MT= 53 (n,n') Level  
Legendre Coefficients

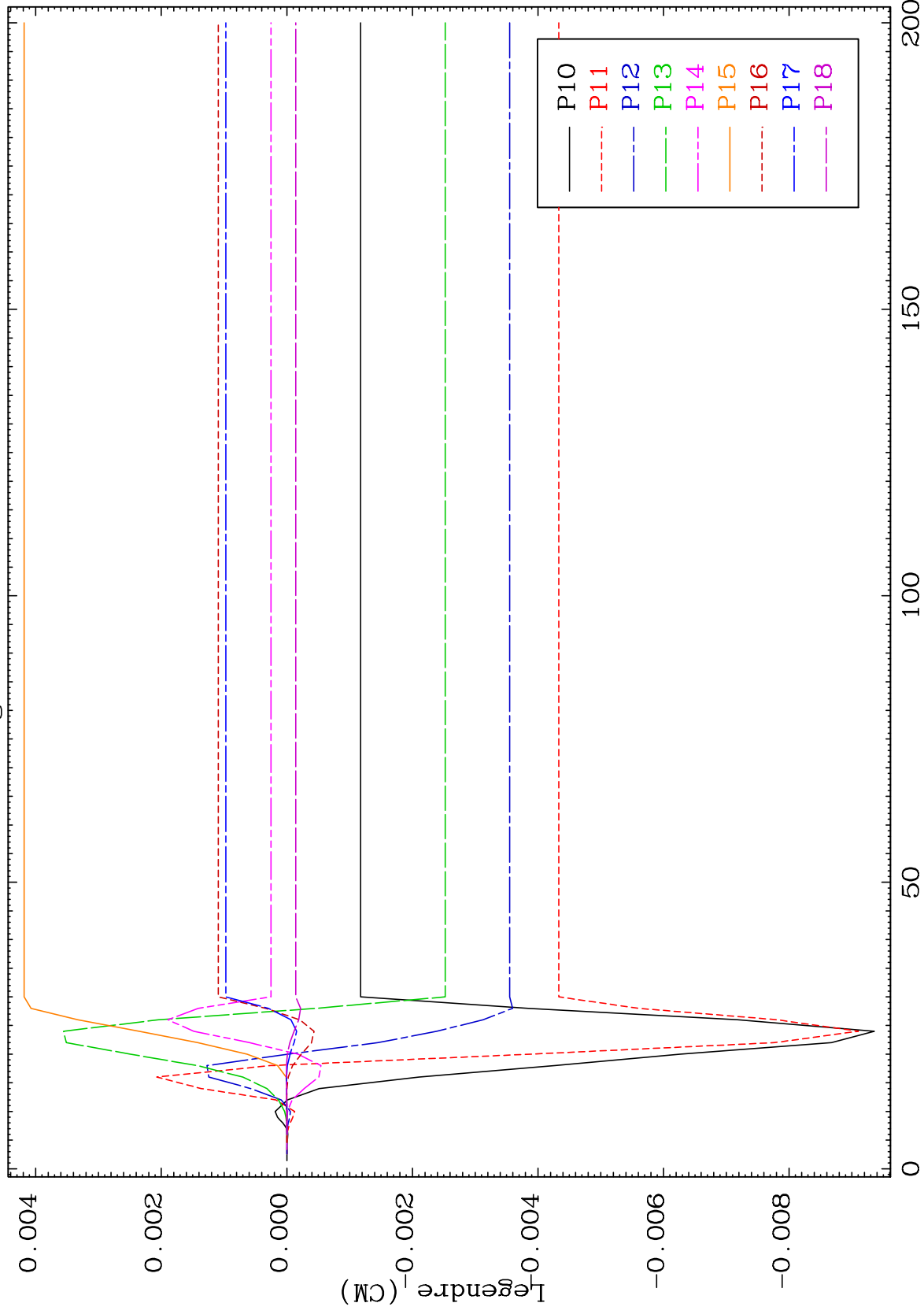
60-Nd-130



MAT 5989

MT= 53 (n,n') Level  
Legendre Coefficients

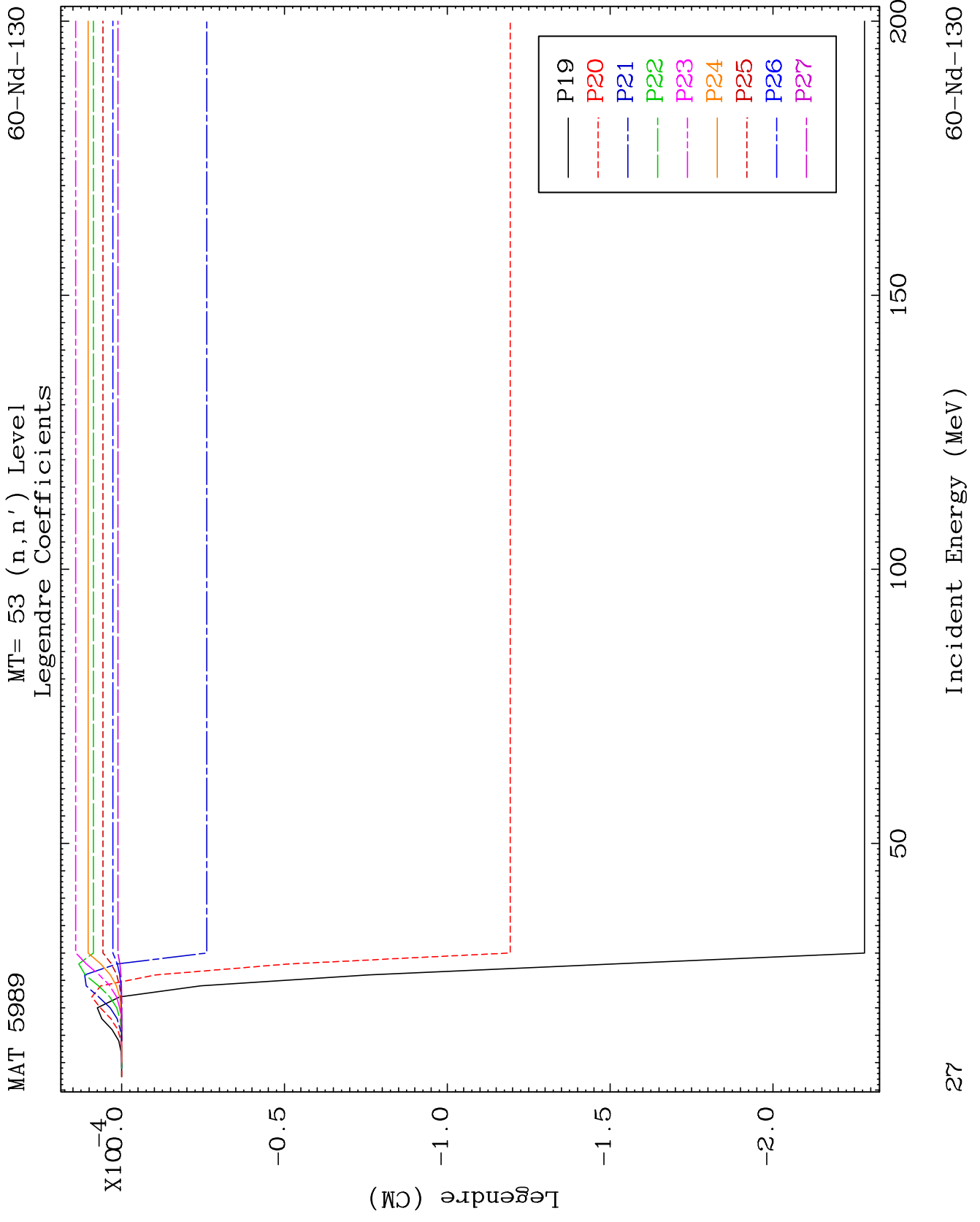
60-Nd-130



26

Incident Energy (MeV)

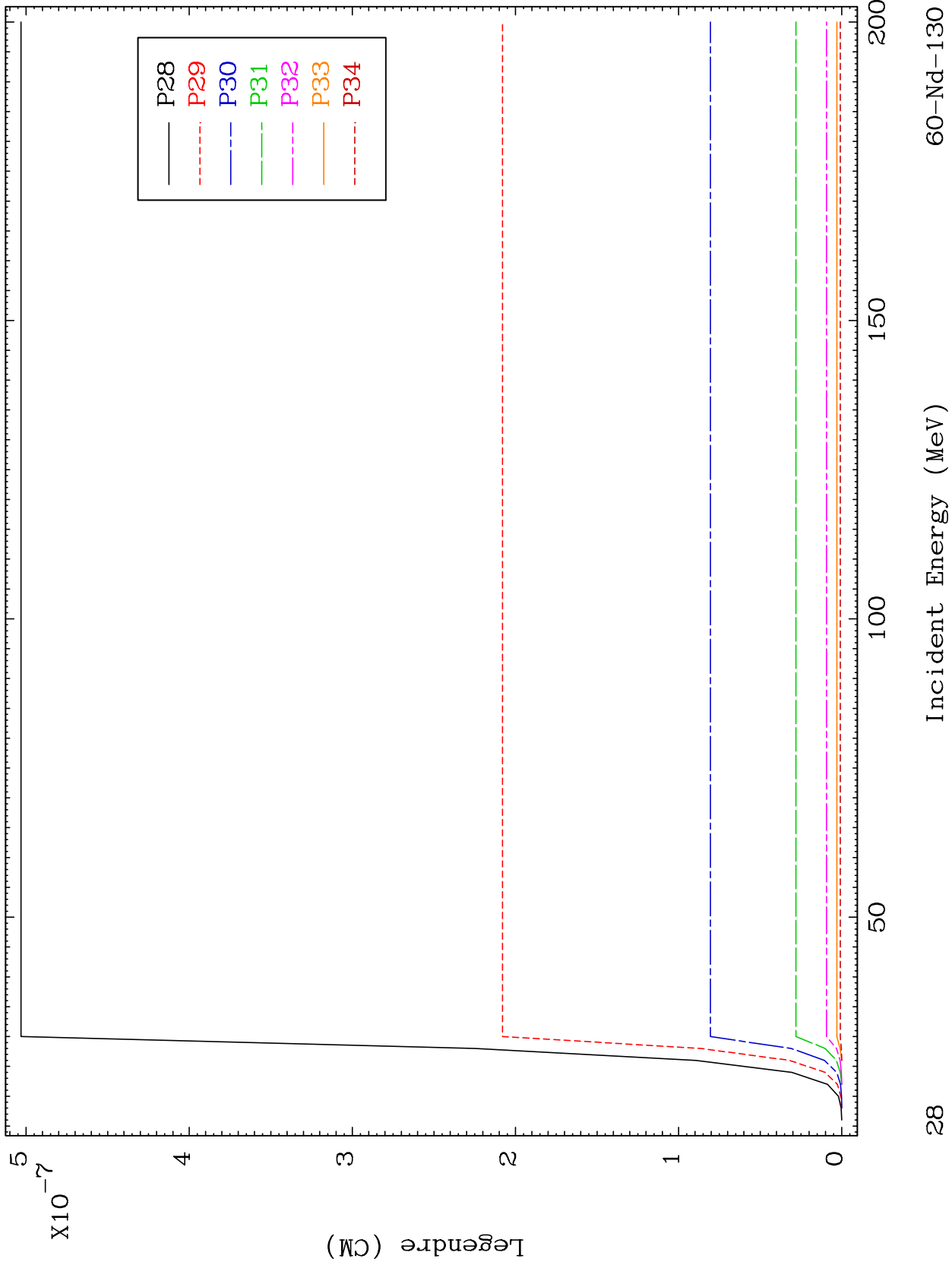
60-Nd-130



MAT 5989

MT= 53 (n,n') Level  
Legendre Coefficients

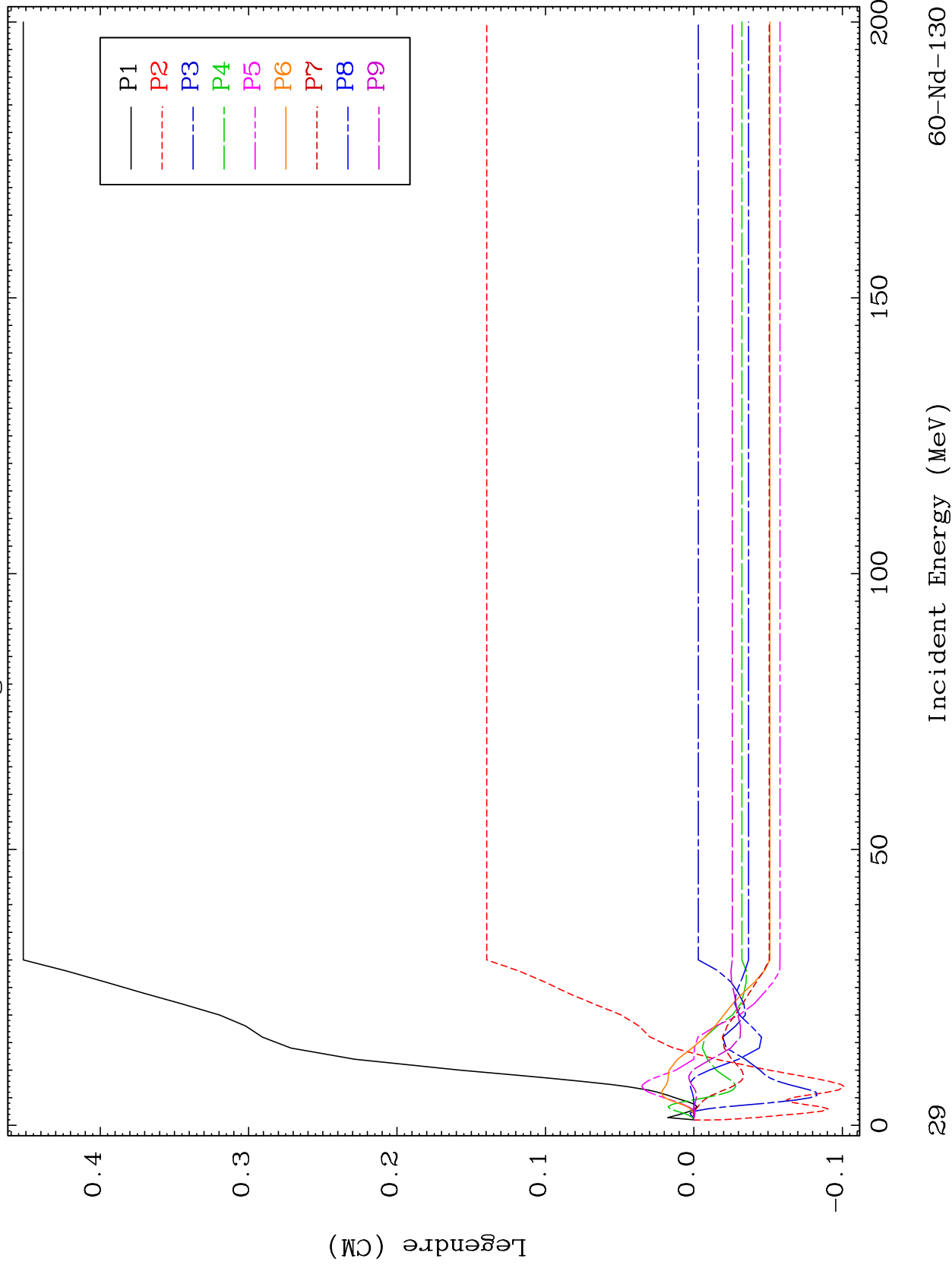
60-Nd-130



MAT 5989

MT= 54 (n,n') Level  
Legendre Coefficients

60-Nd-130



29

Incident Energy (MeV)

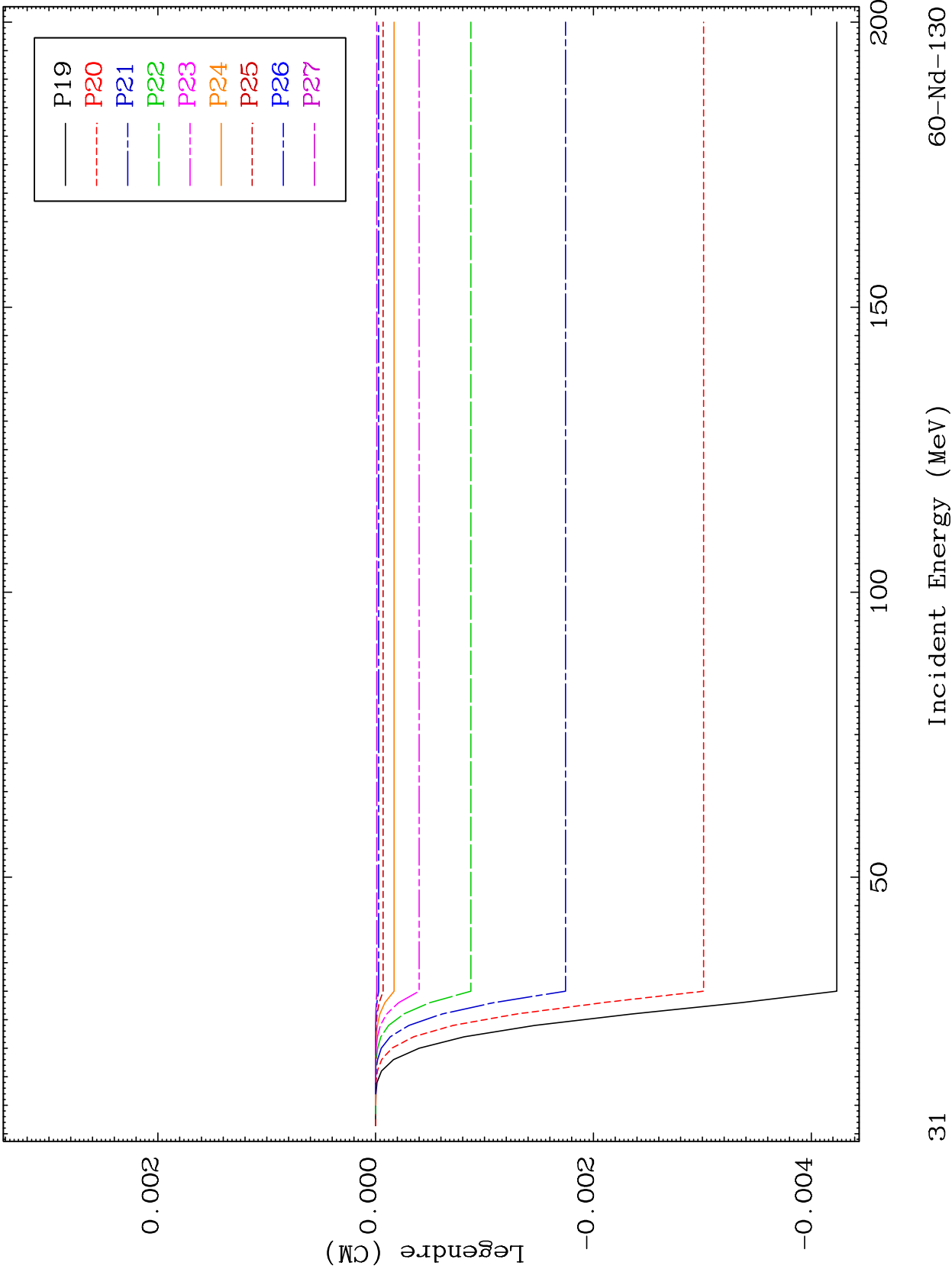
60-Nd-130



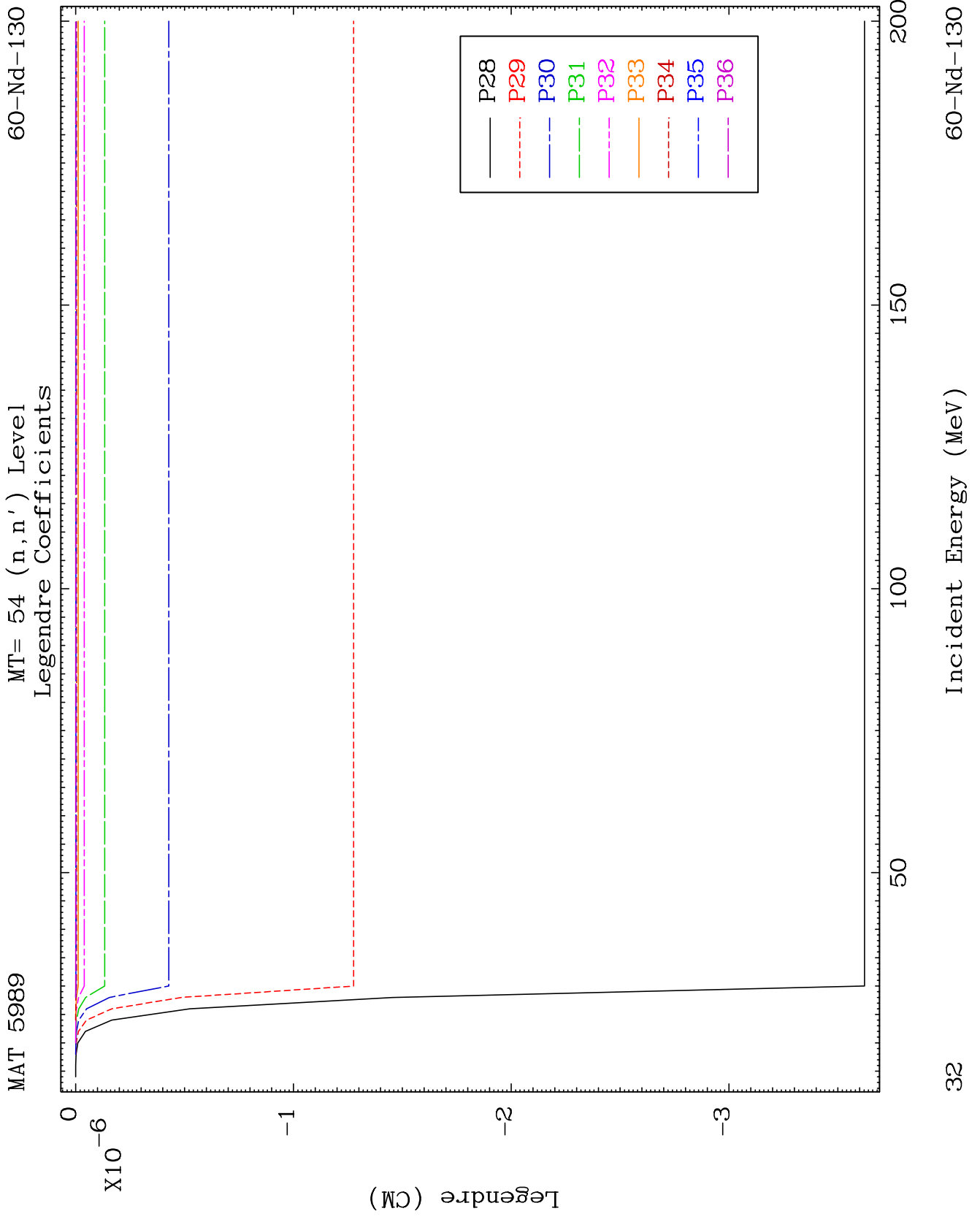
MAT 5989

MT= 54 (n,n') Level  
Legendre Coefficients

60-Nd-130



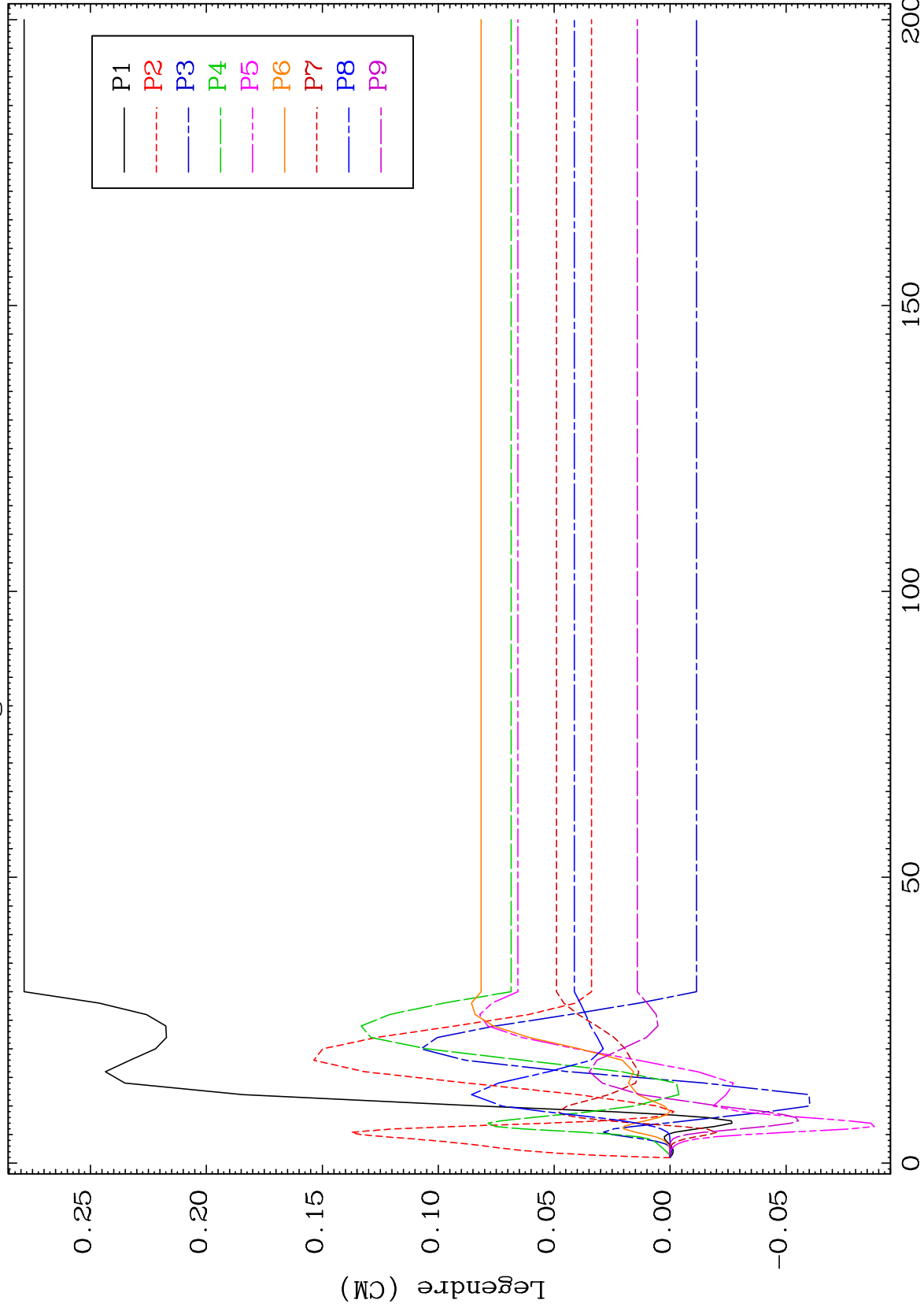




MAT 5989

MT= 55 (n,n') Level  
Legendre Coefficients

60-Nd-130



33

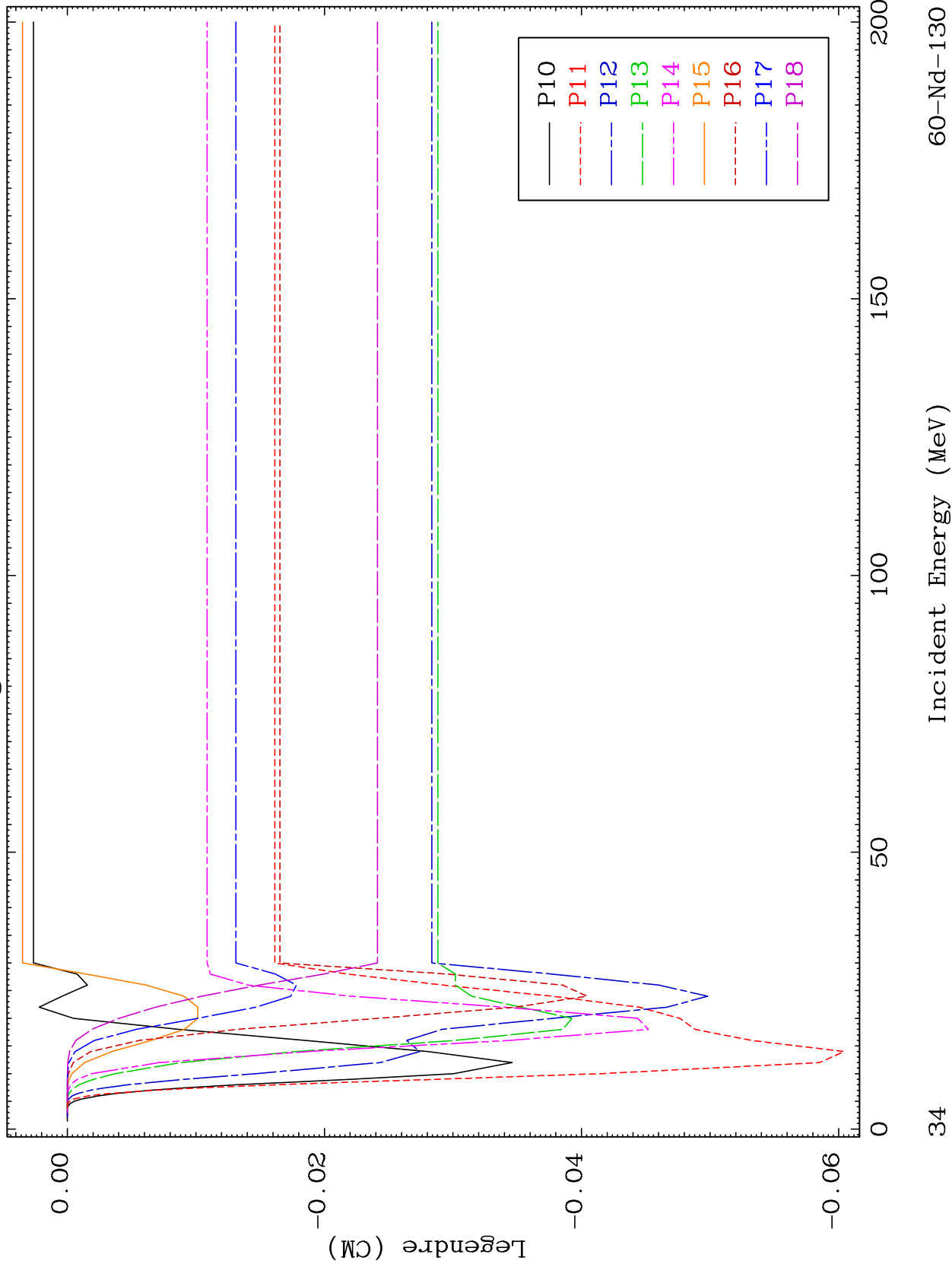
Incident Energy (MeV)

60-Nd-130

MAT 5989

60-Nd-130

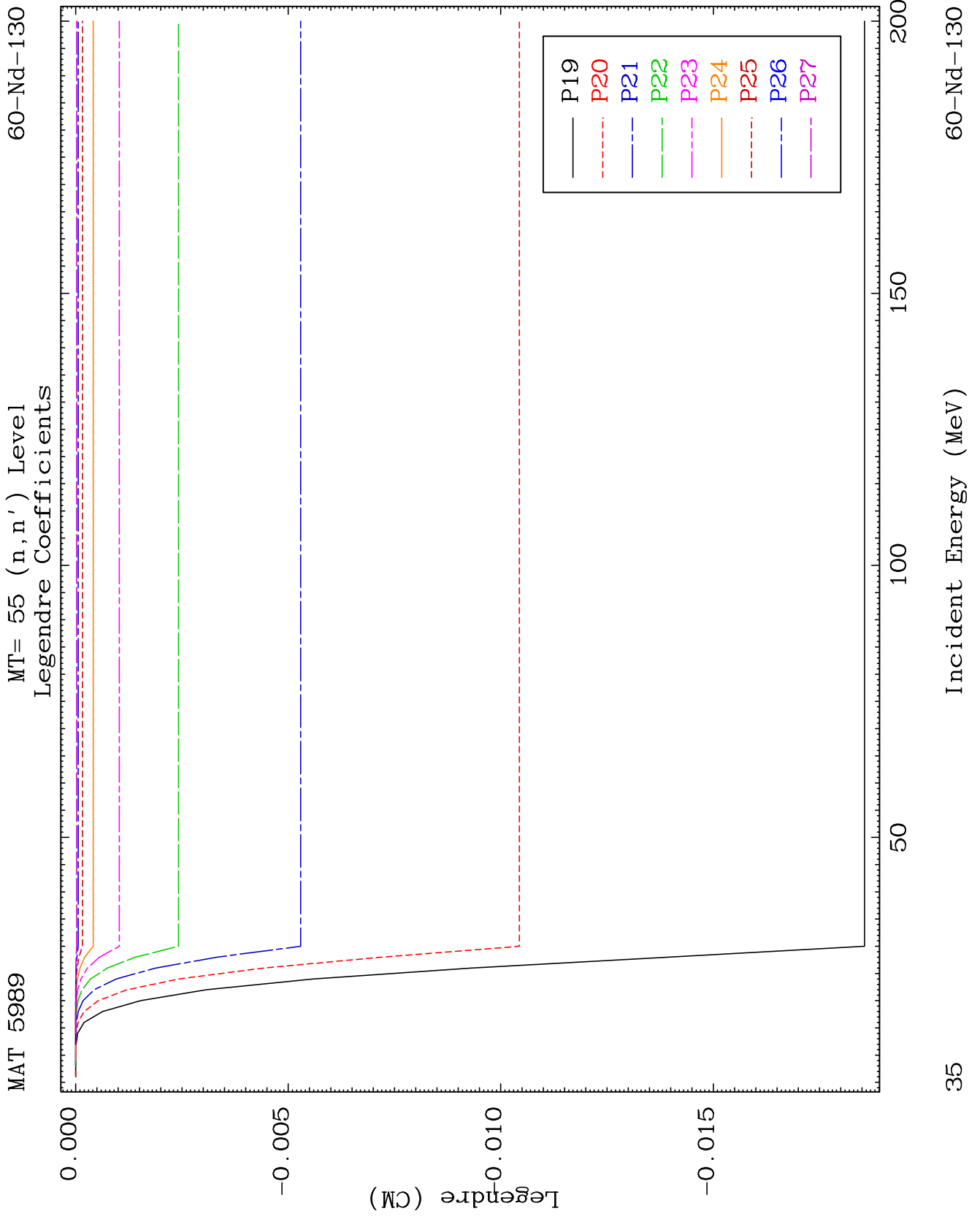
MT= 55 (n,n') Level  
Legendre Coefficients

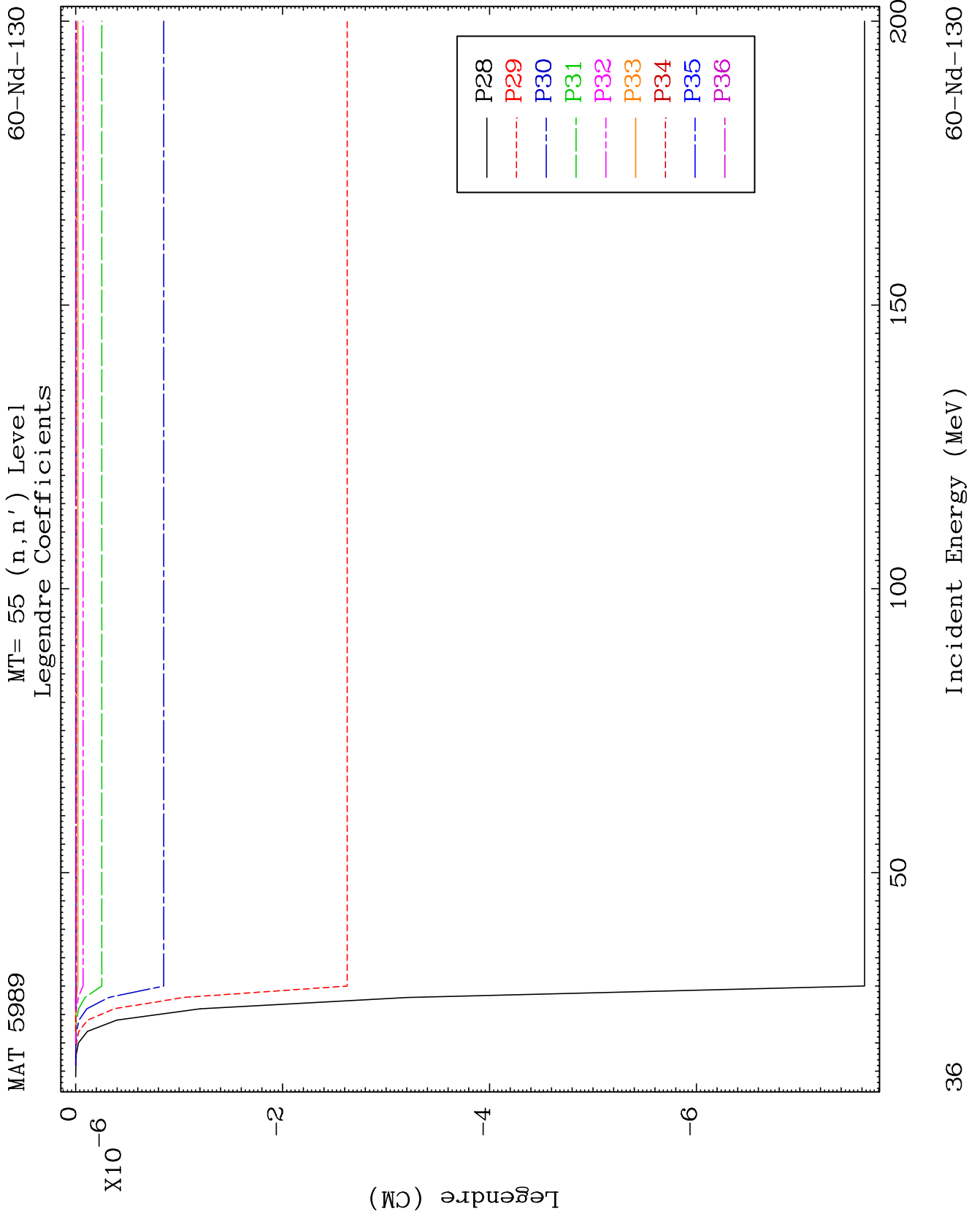


34

Incident Energy (MeV)

60-Nd-130

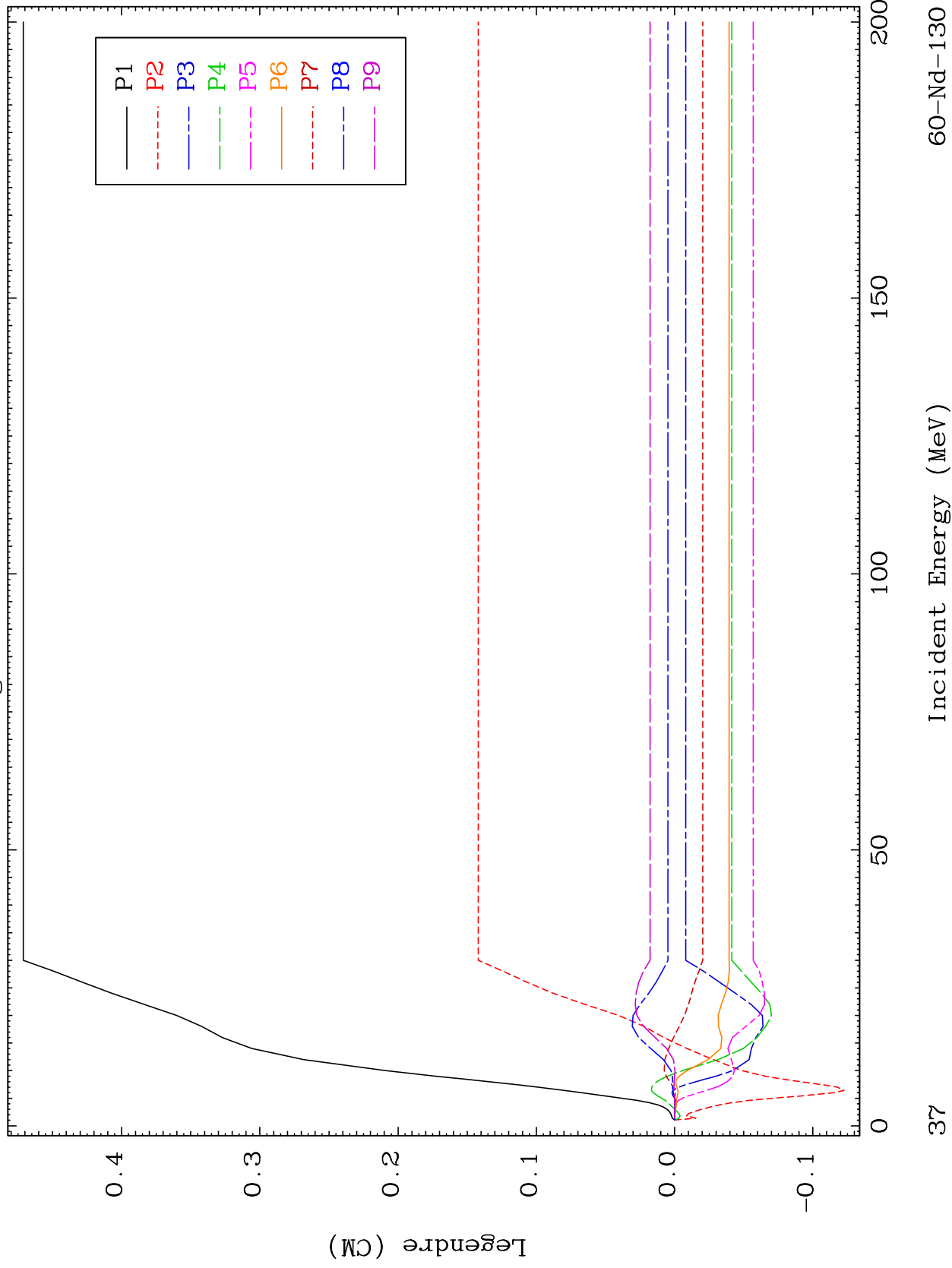




MAT 5989

MT= 56 (n,n') Level  
Legendre Coefficients

60-Nd-130



60-Nd-130

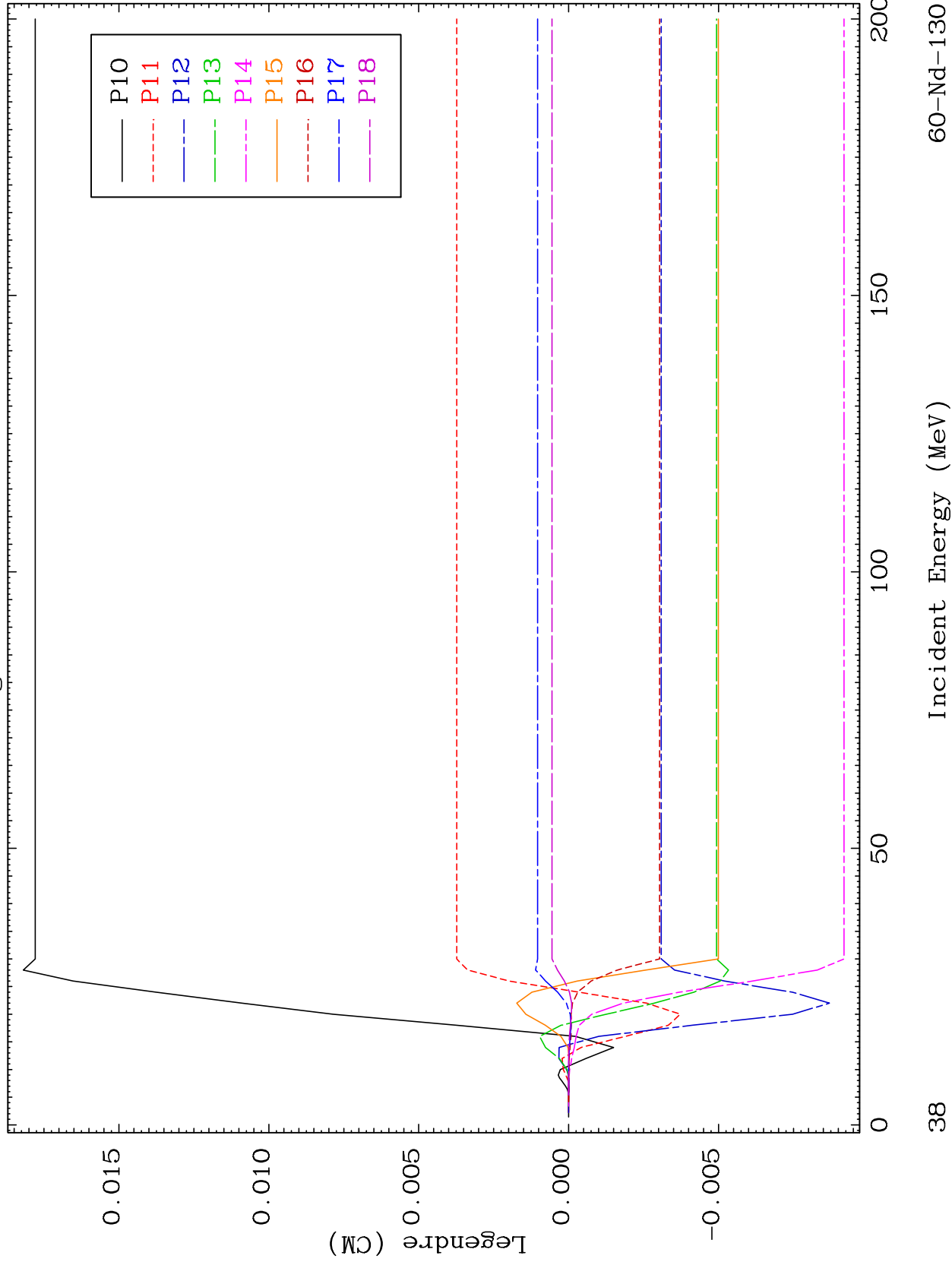
Incident Energy (MeV)

37

MAT 5989

MT= 56 (n,n') Level  
Legendre Coefficients

60-Nd-130



60-Nd-130

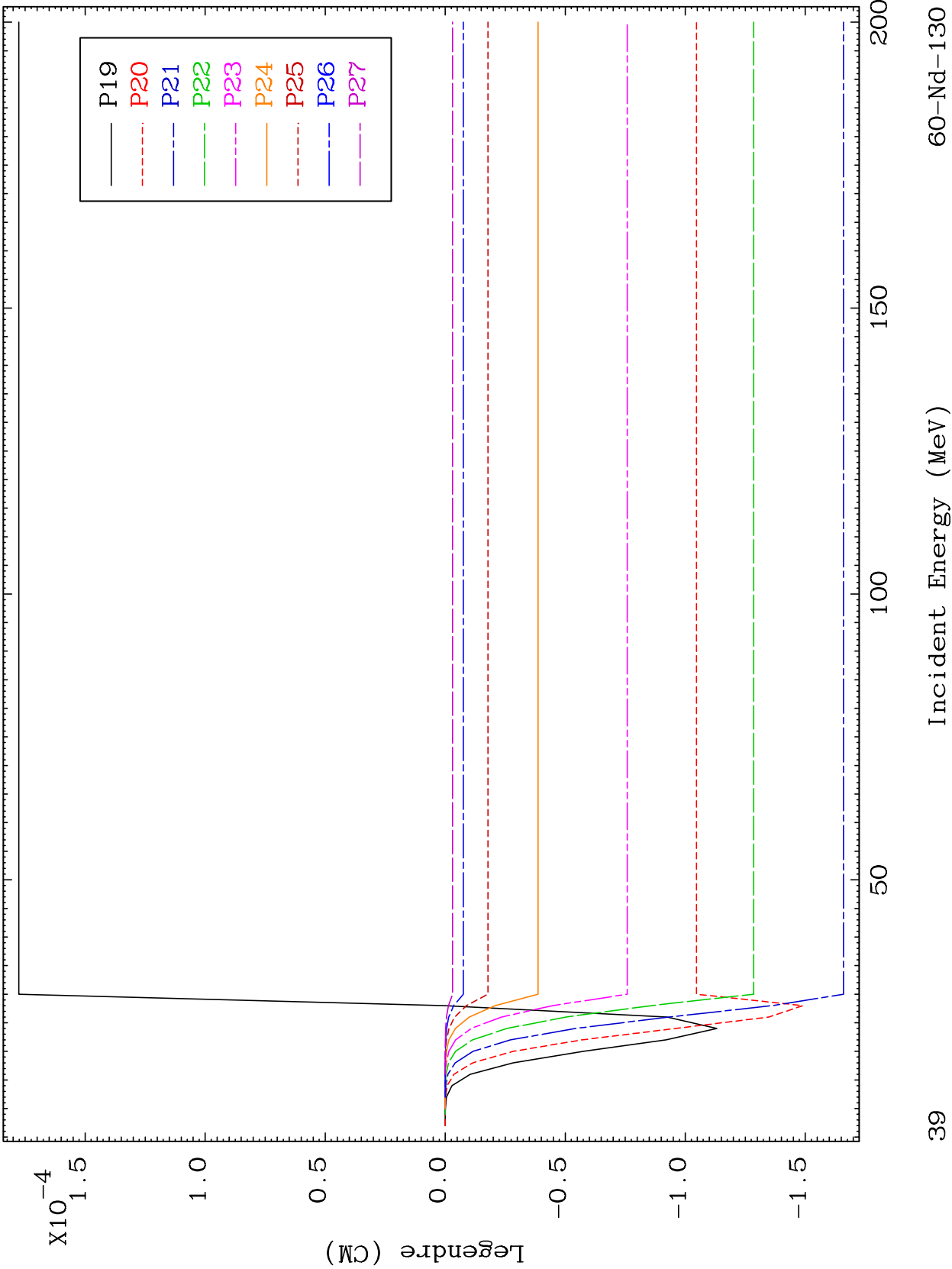
Incident Energy (MeV)

38

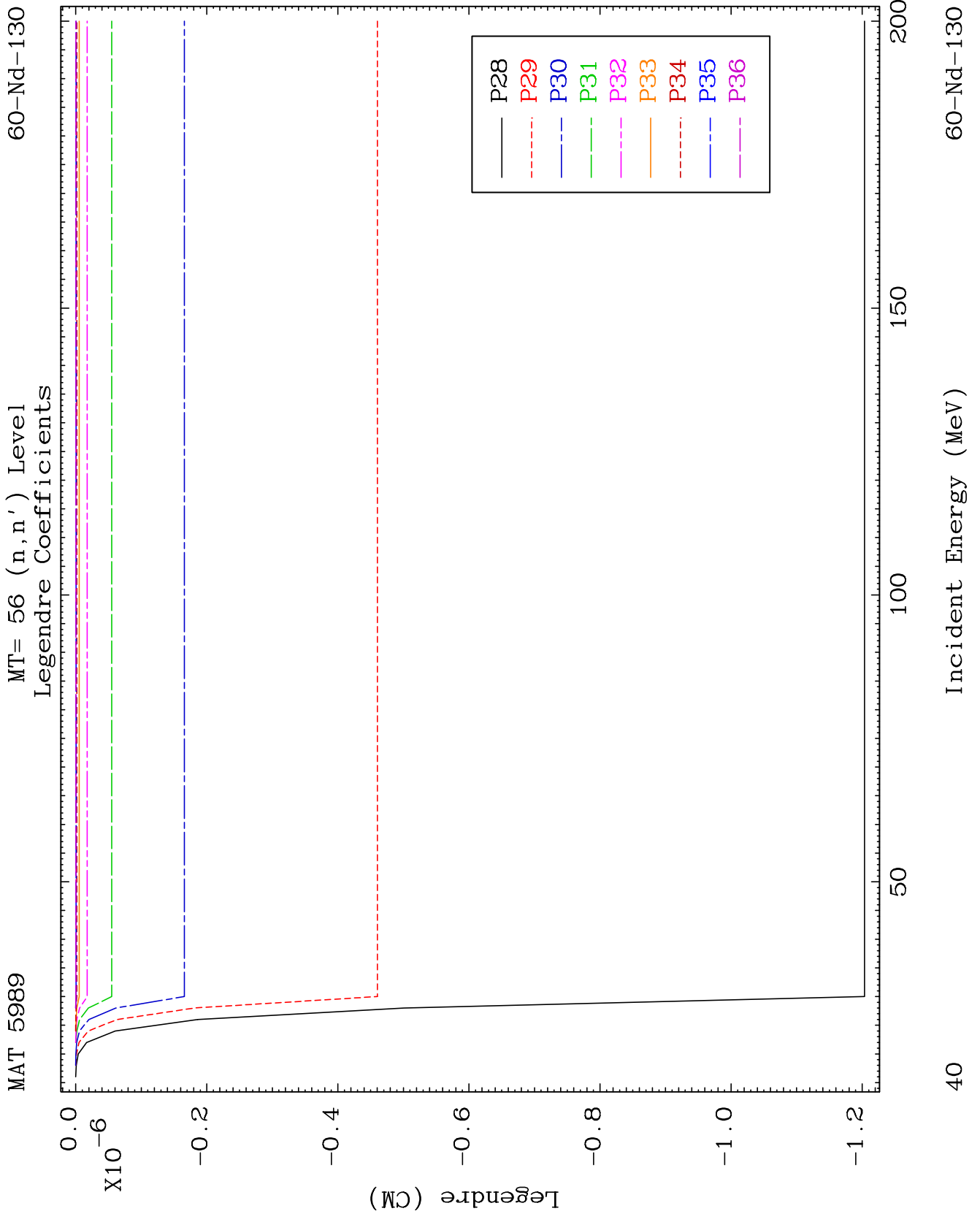
MAT 5989

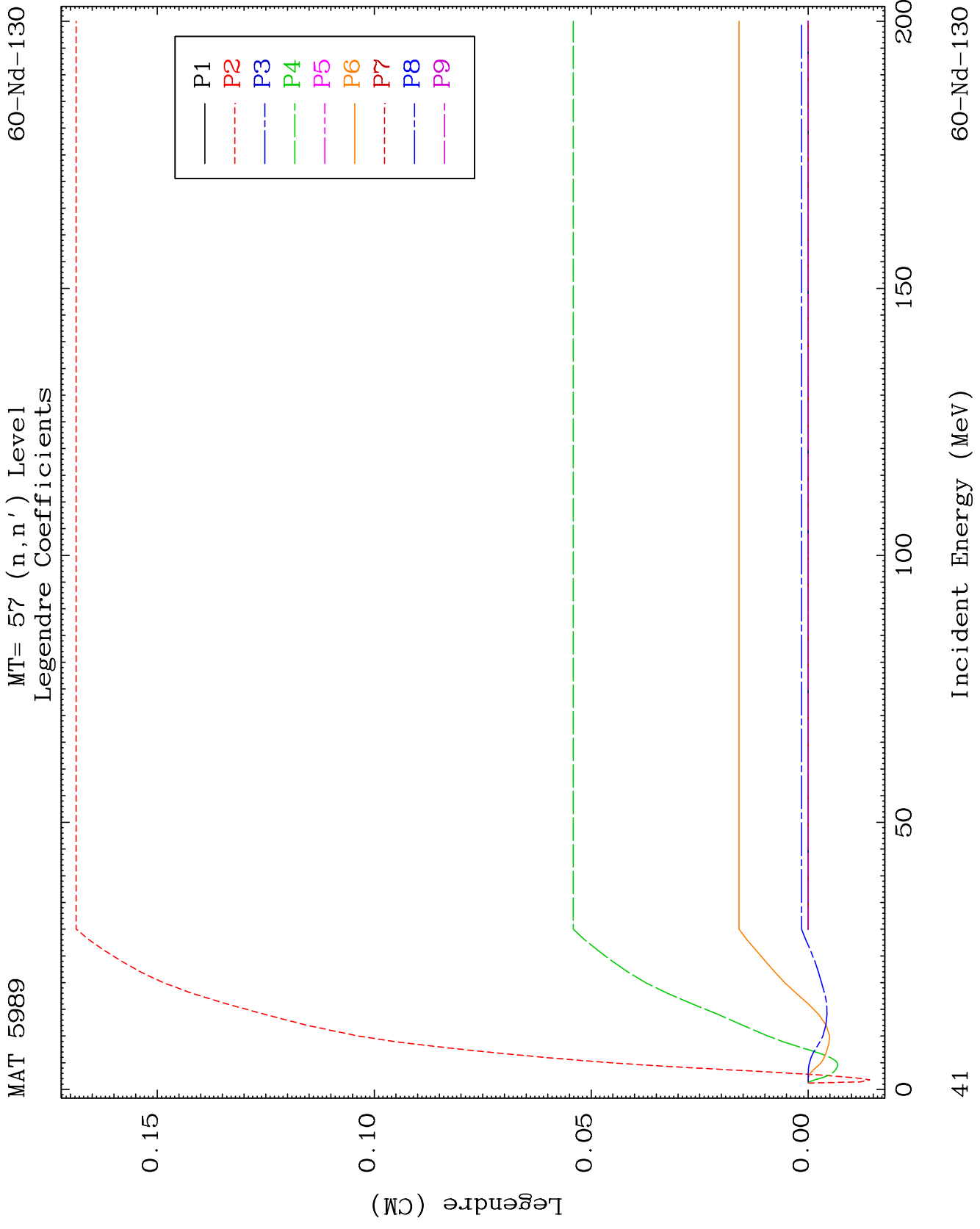
MT= 56 (n,n') Level  
Legendre Coefficients

60-Nd-130





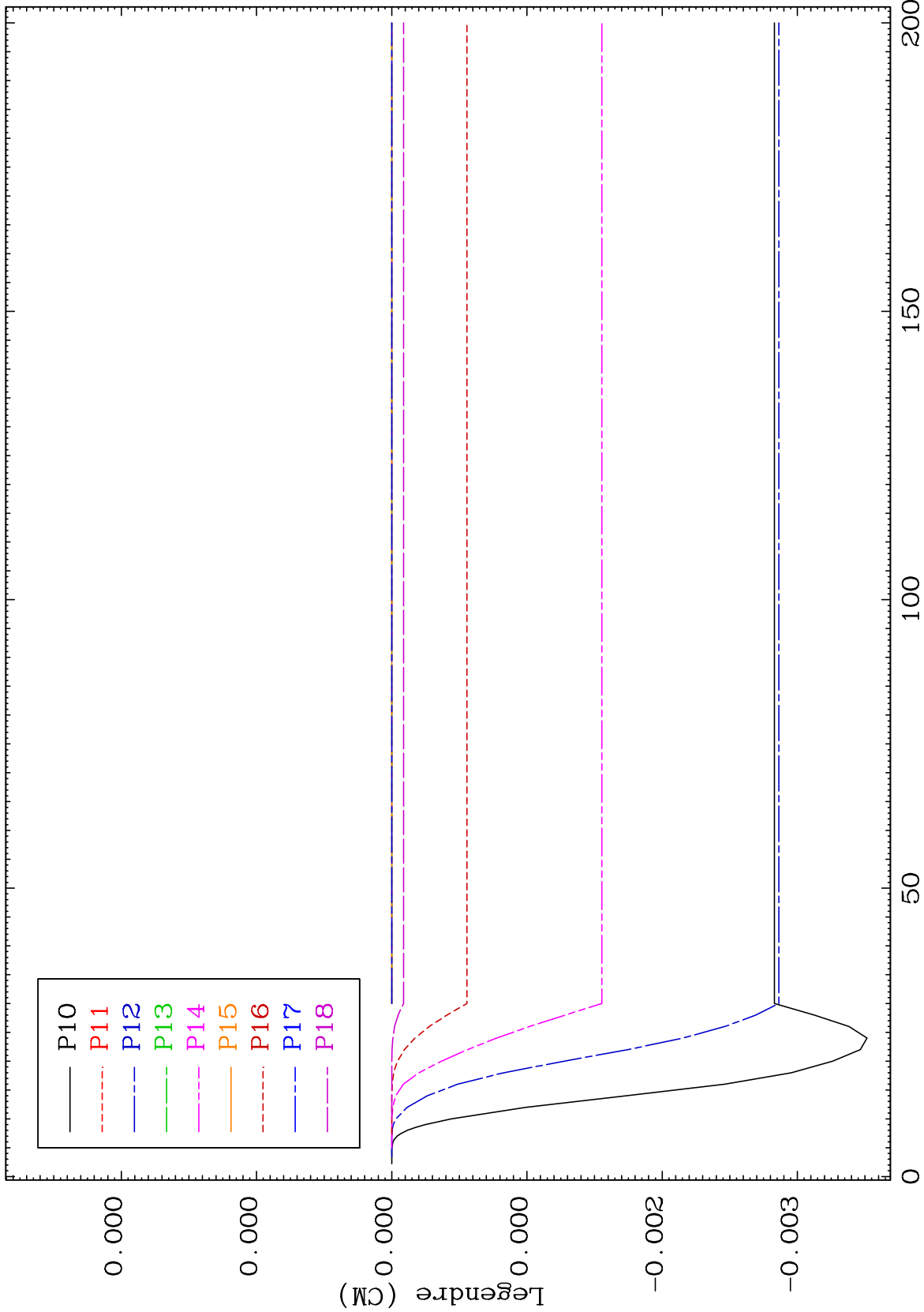




MAT 5989

MT= 57 (n,n') Level  
Legendre Coefficients

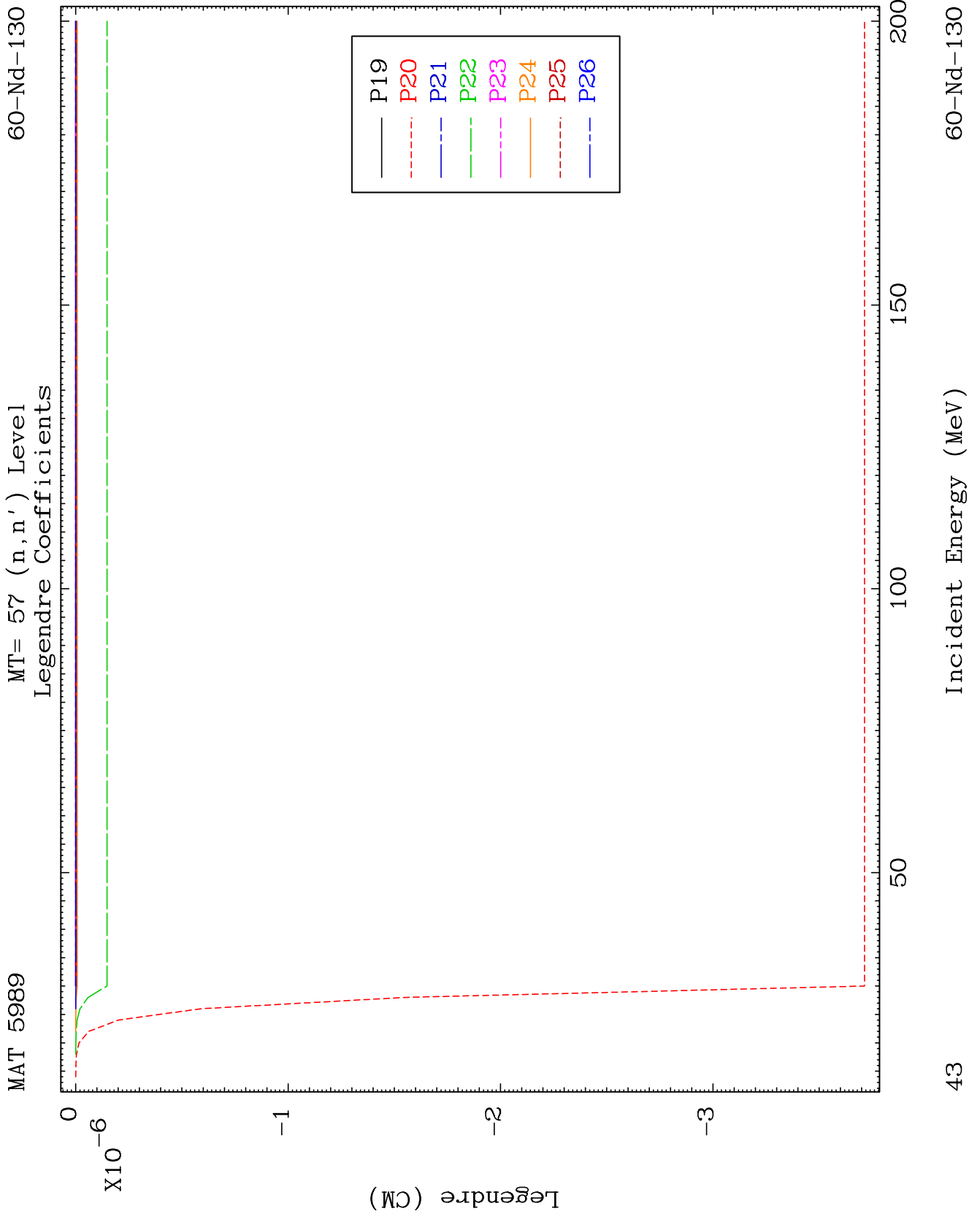
60-Nd-130



42

Incident Energy (MeV)

60-Nd-130

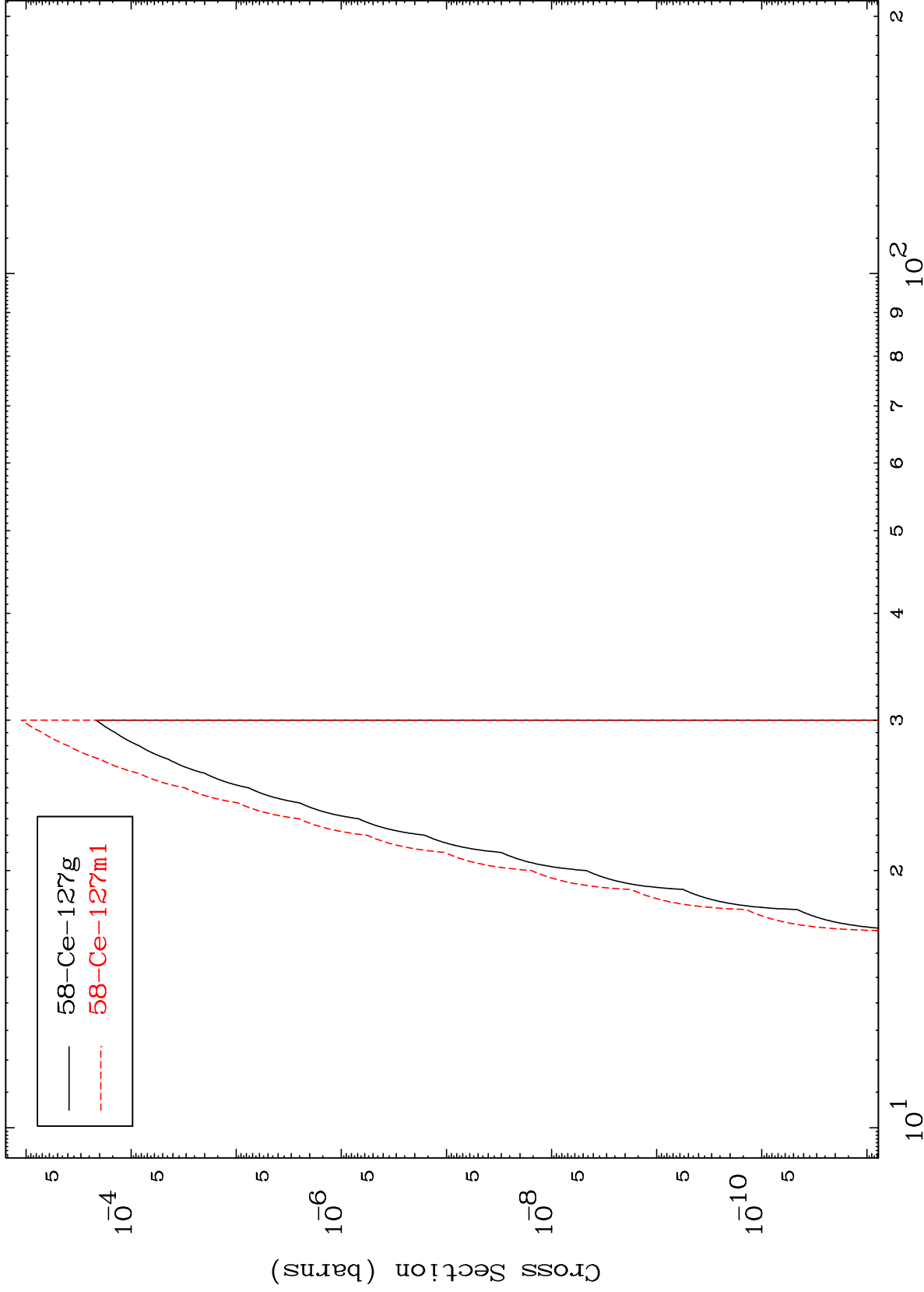


MAT 5989

(n,n') He-3

60-Nd-130

Radionuclide Production Cross Section



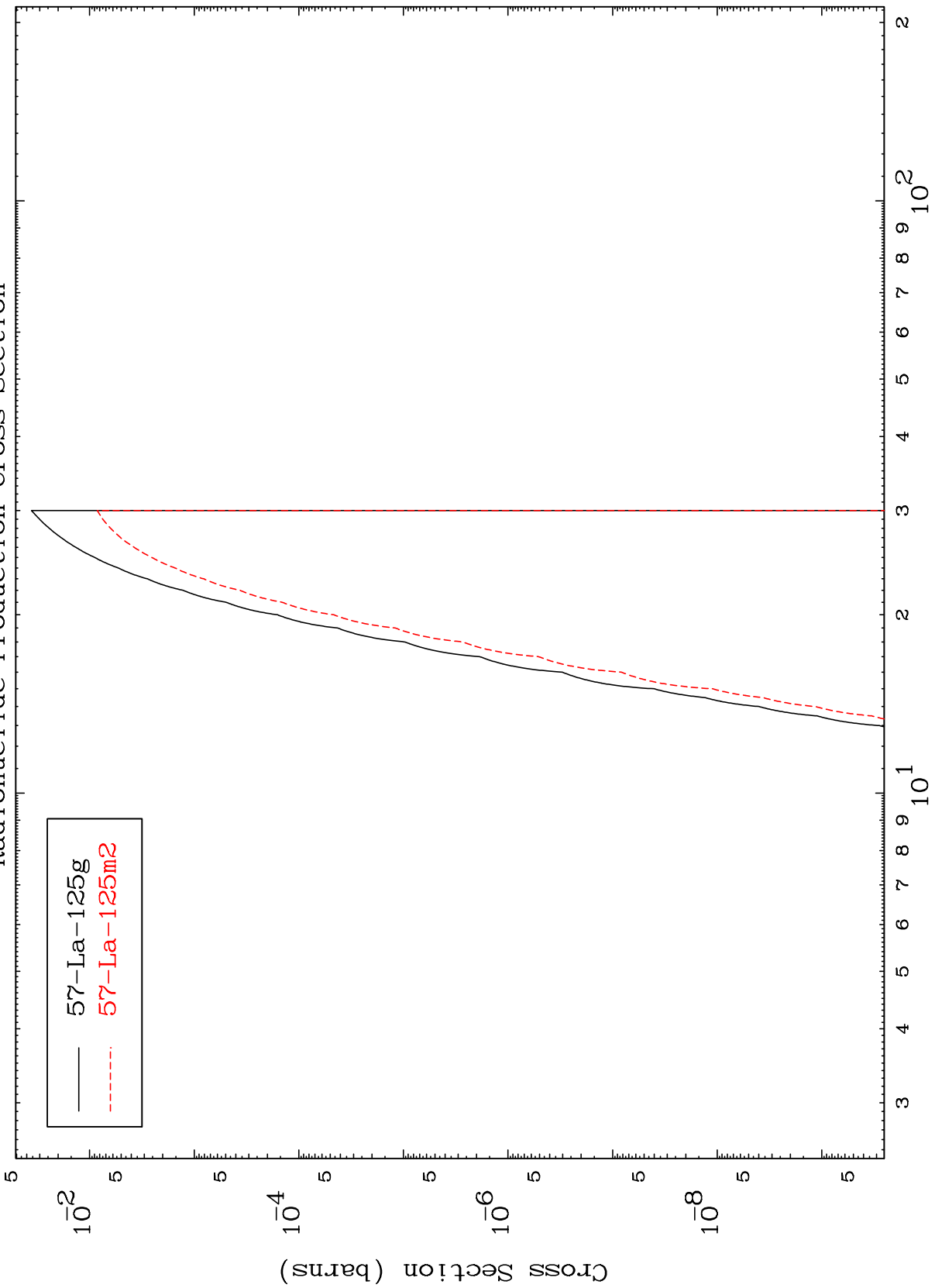
58-Ce-127g  
58-Ce-127m1

MAT 5989

(n,n') p  $\alpha$

60-Nd-130

Radionuclide Production Cross Section



57-La-125g  
57-La-125m2

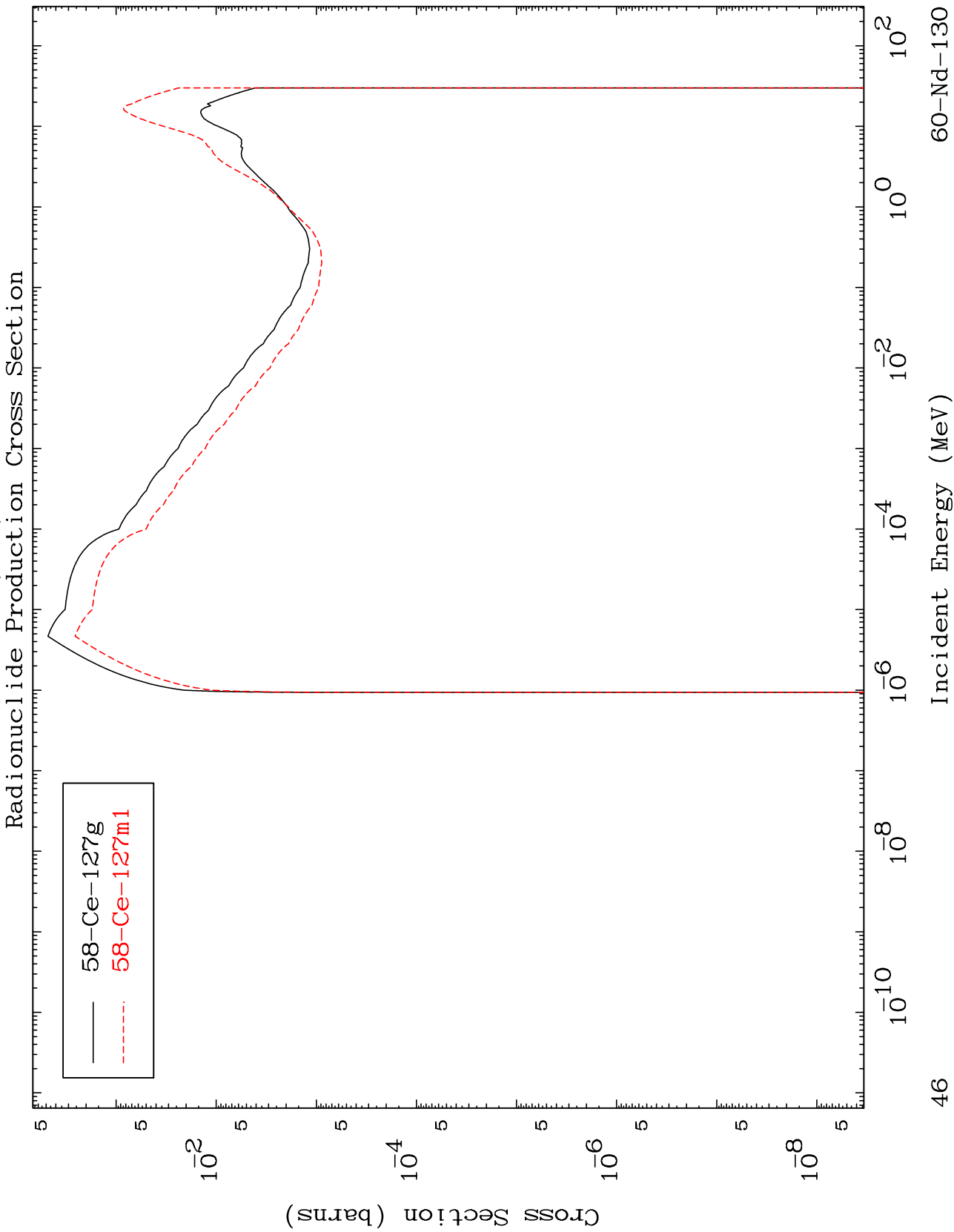
45

Incident Energy (MeV)

60-Nd-130

MAT 5989

60-Nd-130



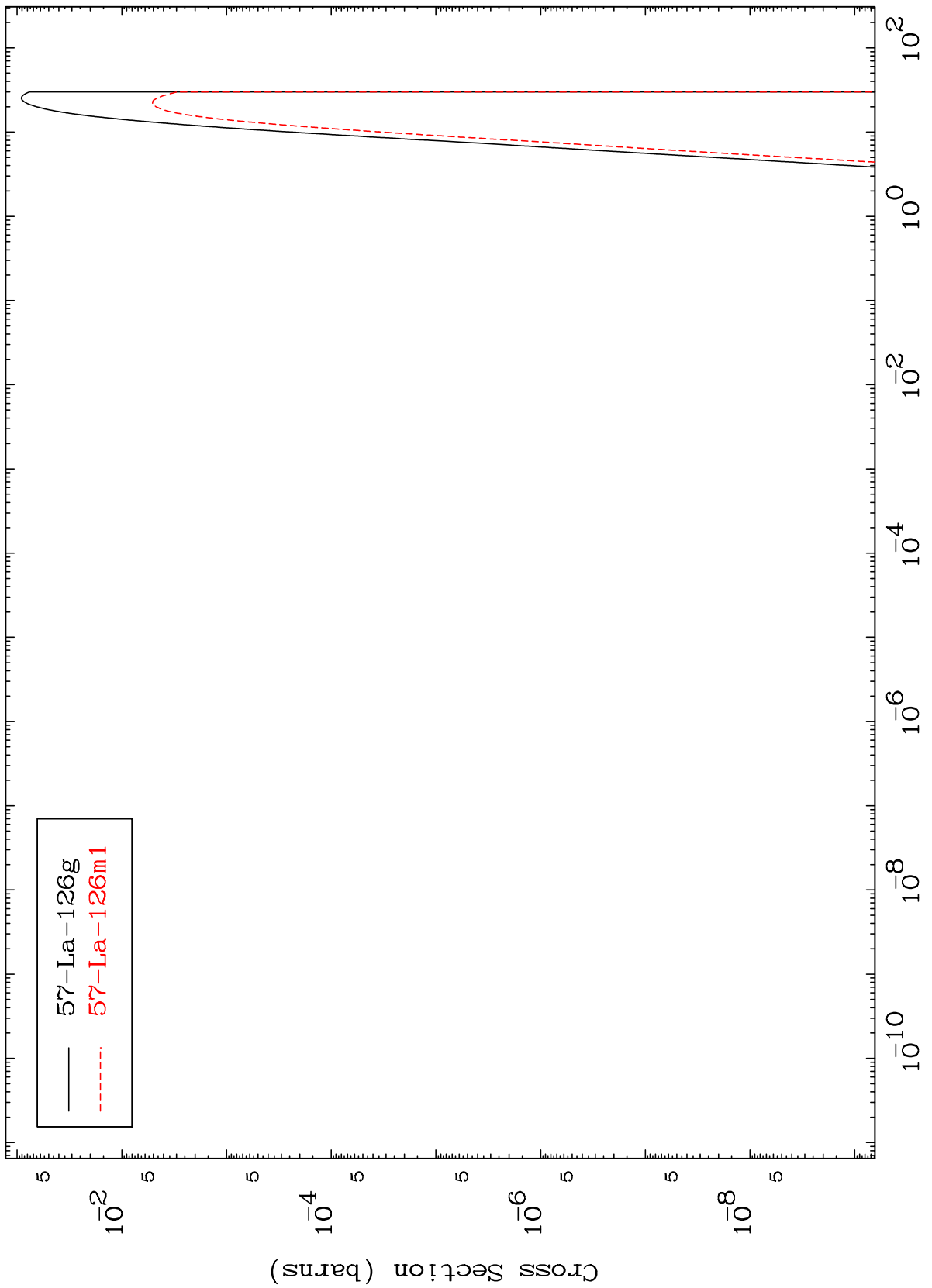
— 58-Ce-127g  
- - - 58-Ce-127m1

MAT 5989

(n,p)  $\alpha$

60-Nd-130

Radionuclide Production Cross Section



47

Incident Energy (MeV)

60-Nd-130

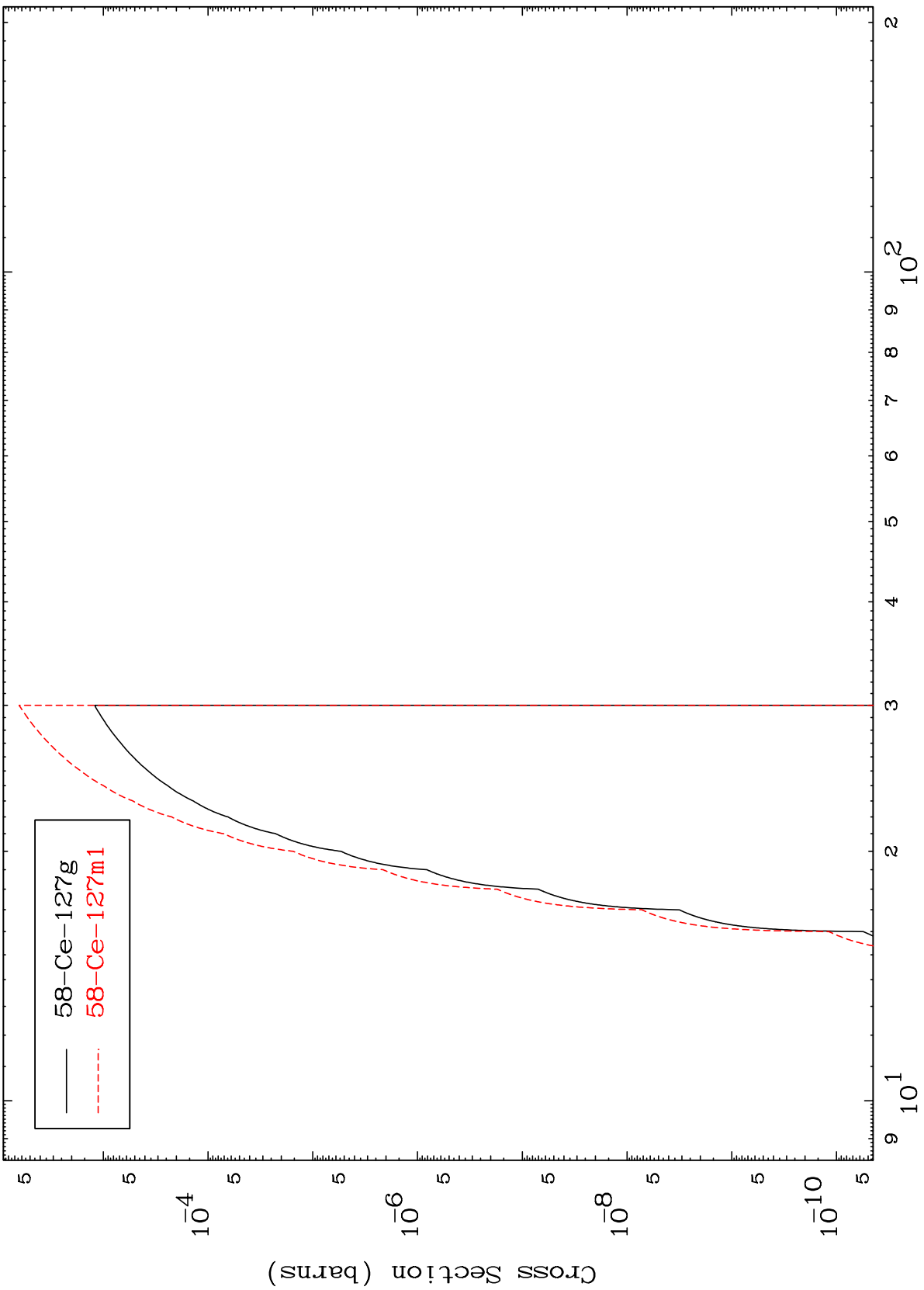


MAT 5989

(n,p) t

60-Nd-130

Radionuclide Production Cross Section



48

Incident Energy (MeV)

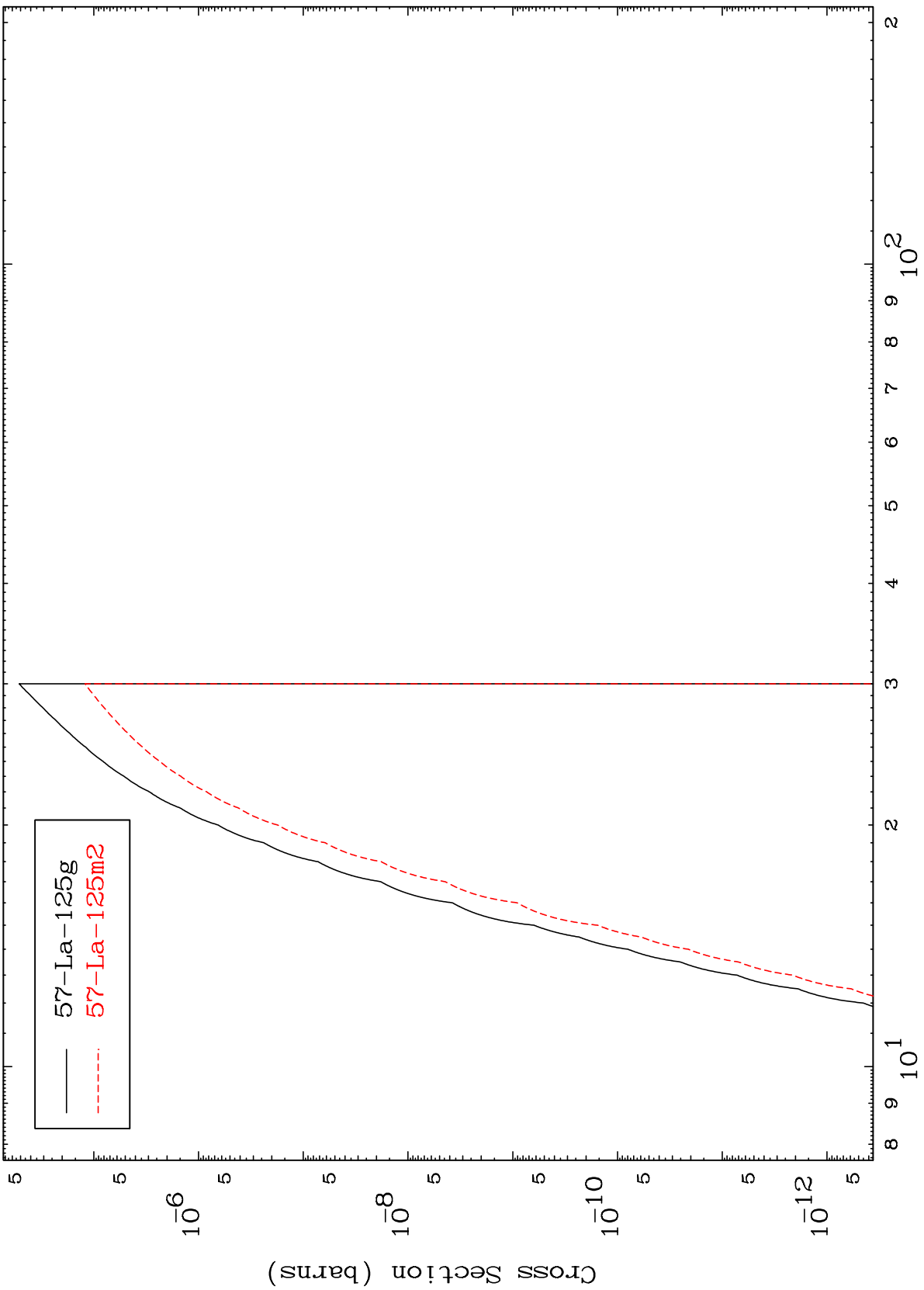
60-Nd-130

MAT 5989

(n,d)  $\alpha$

60-Nd-130

Radionuclide Production Cross Section



49

Incident Energy (MeV)

60-Nd-130