

Program EVALPLOT
(Version 2018-1)

by

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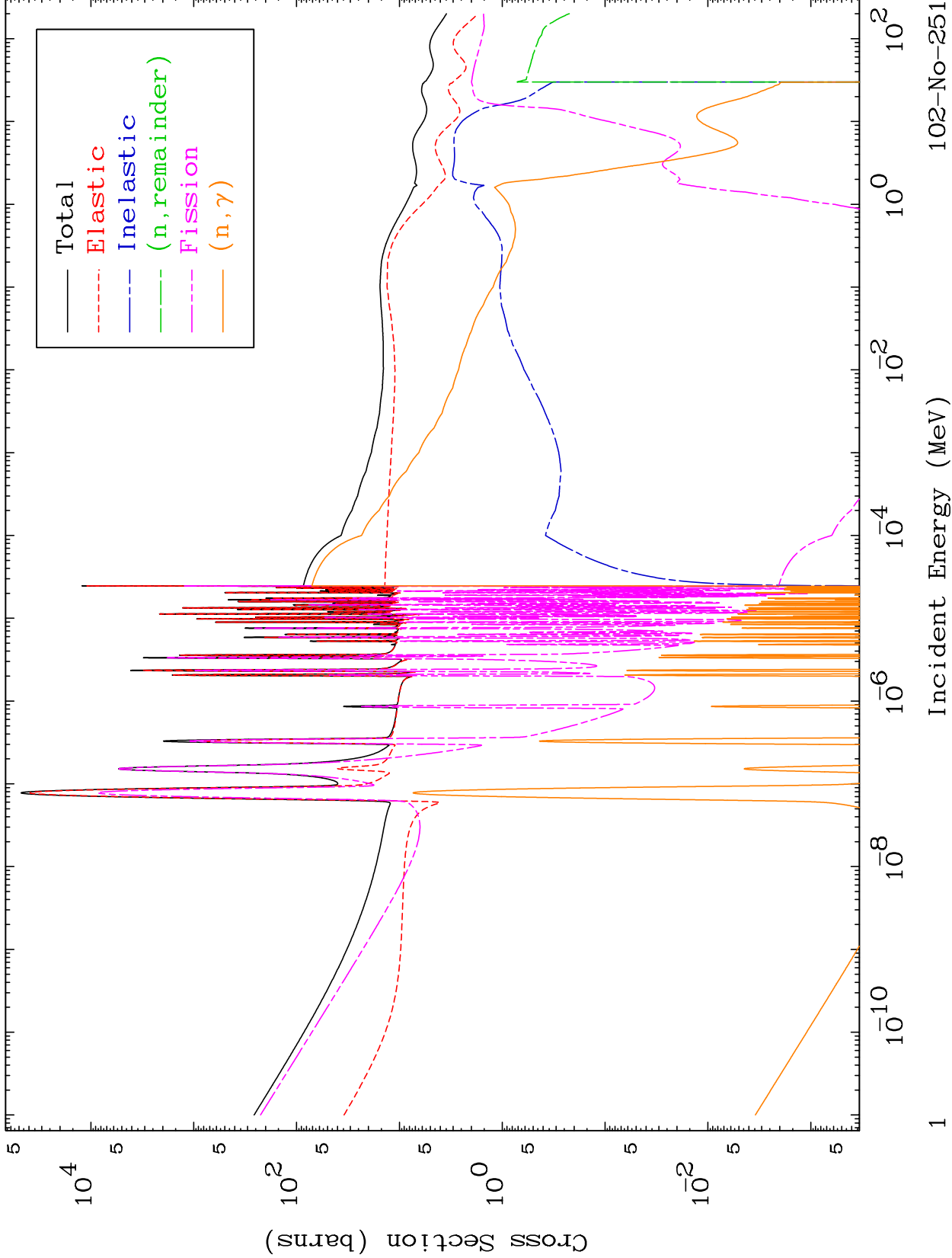
Press Mouse Button to Start

MAT 9974

Major

293 Kelvin Cross Sections

102-No-251

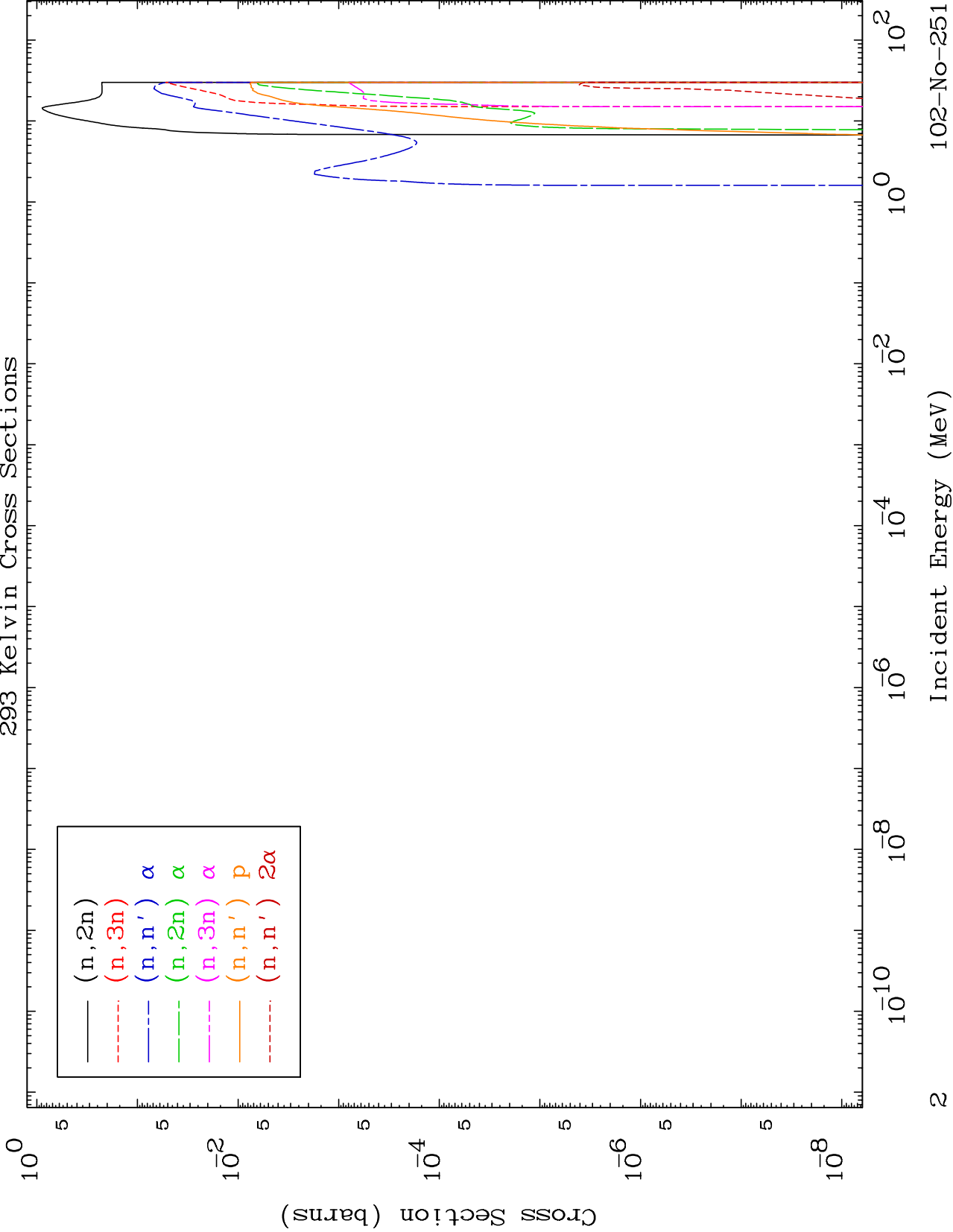


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MAT 9974

Neutron Production
293 Kelvin Cross Sections

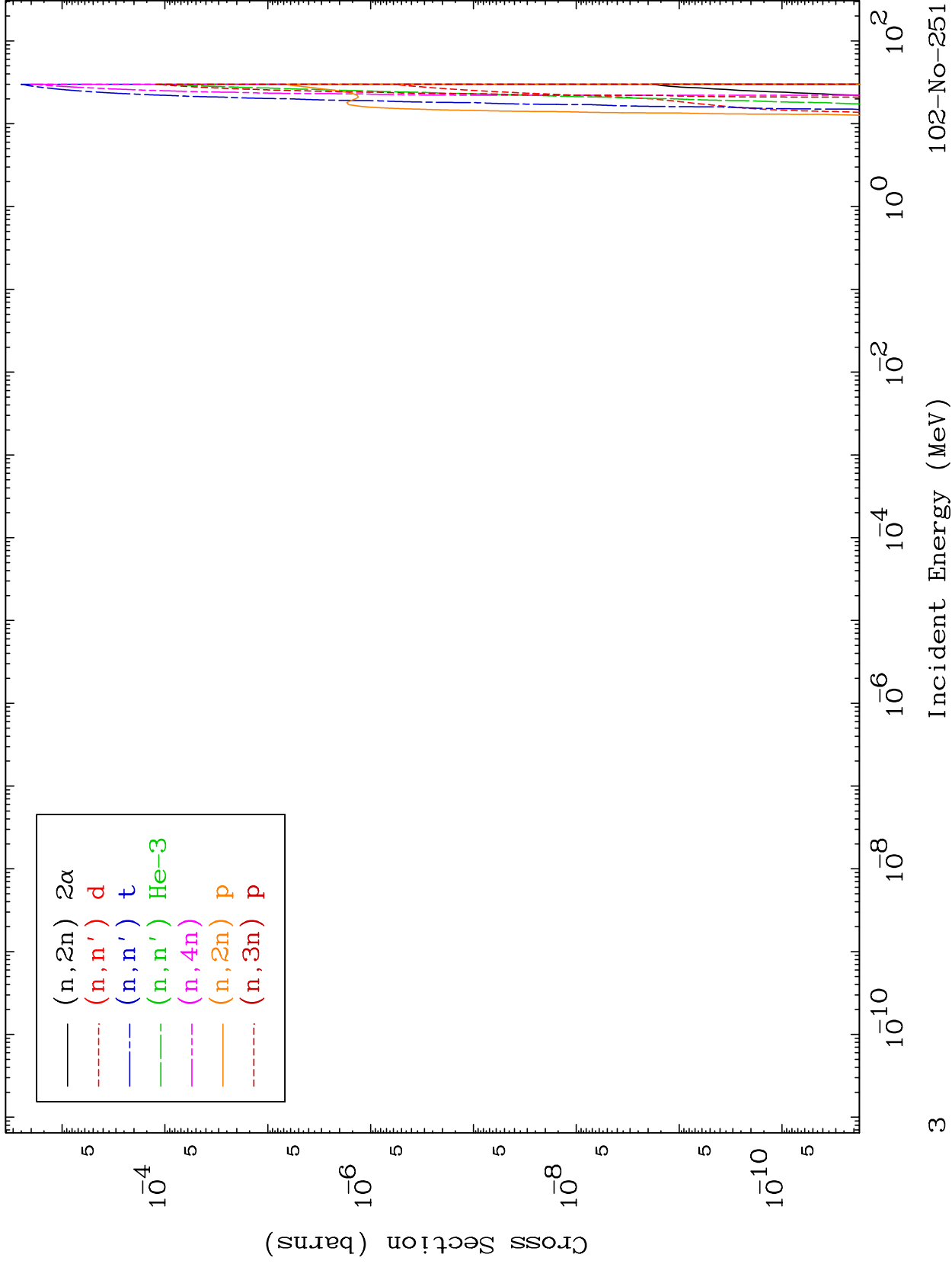
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MAT 9974

Neutron Production
293 Kelvin Cross Sections

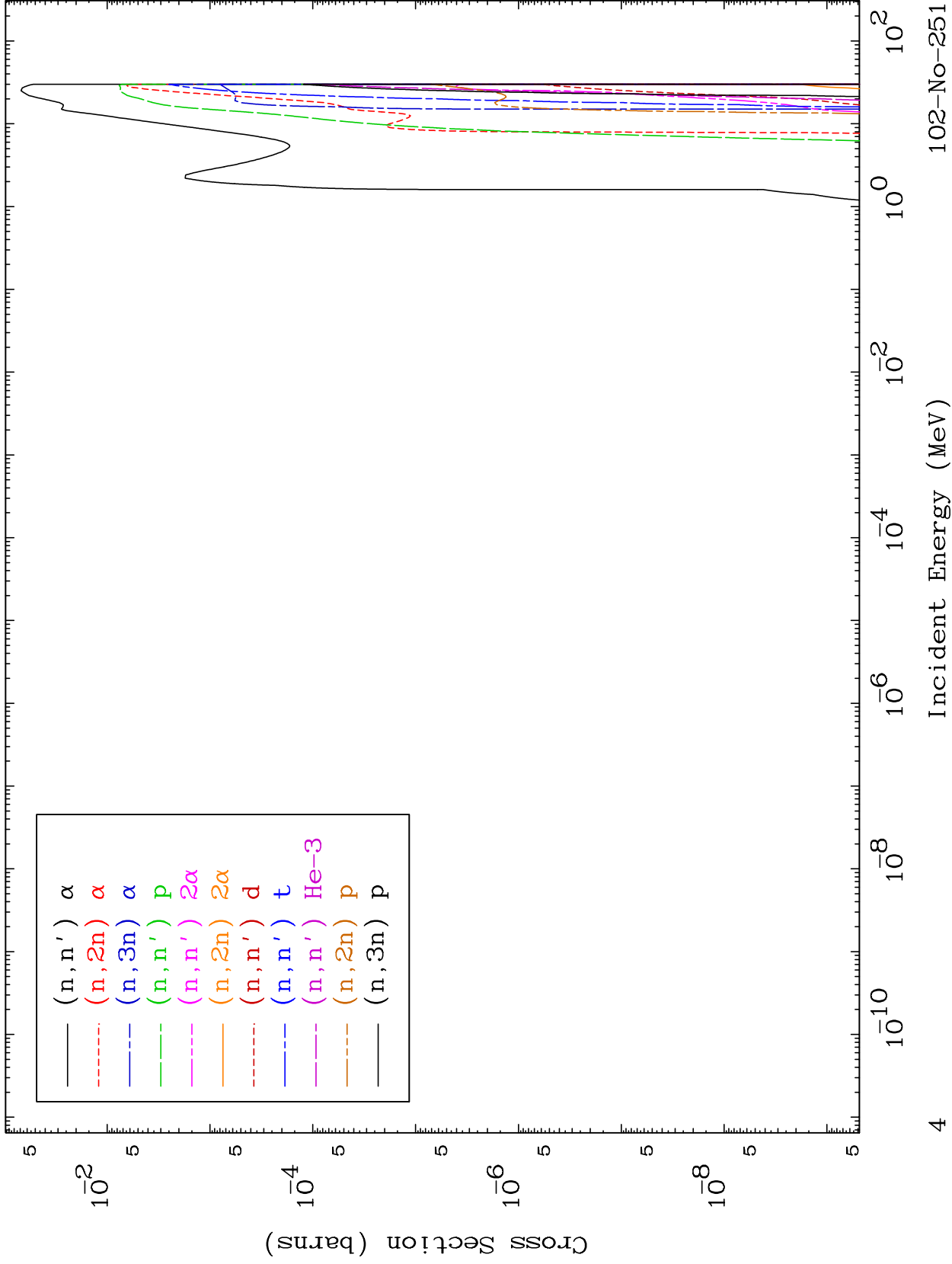
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MAT 9974

Charged Particle
293 Kelvin Cross Sections

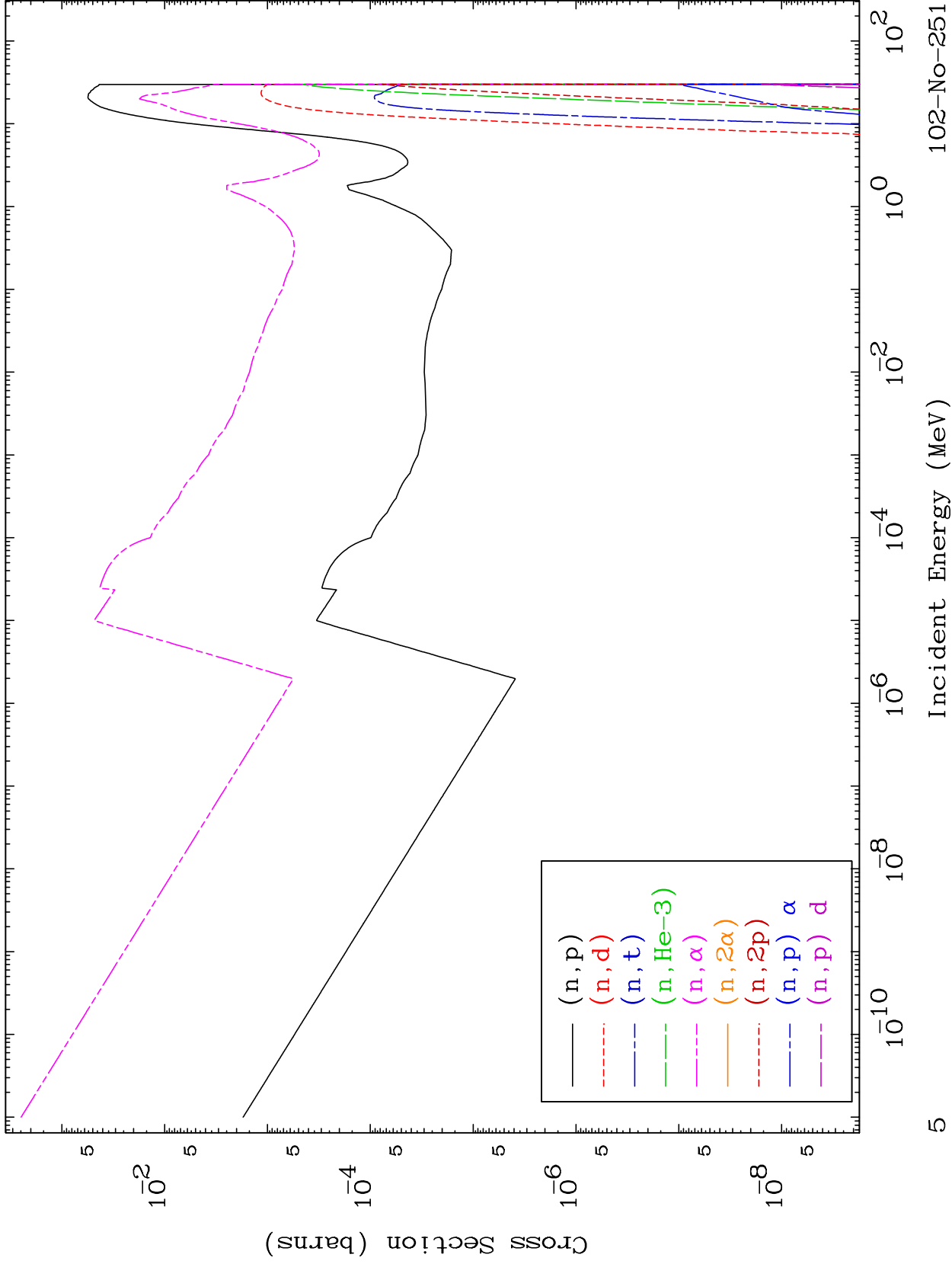
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Charged Particle
293 Kelvin Cross Sections

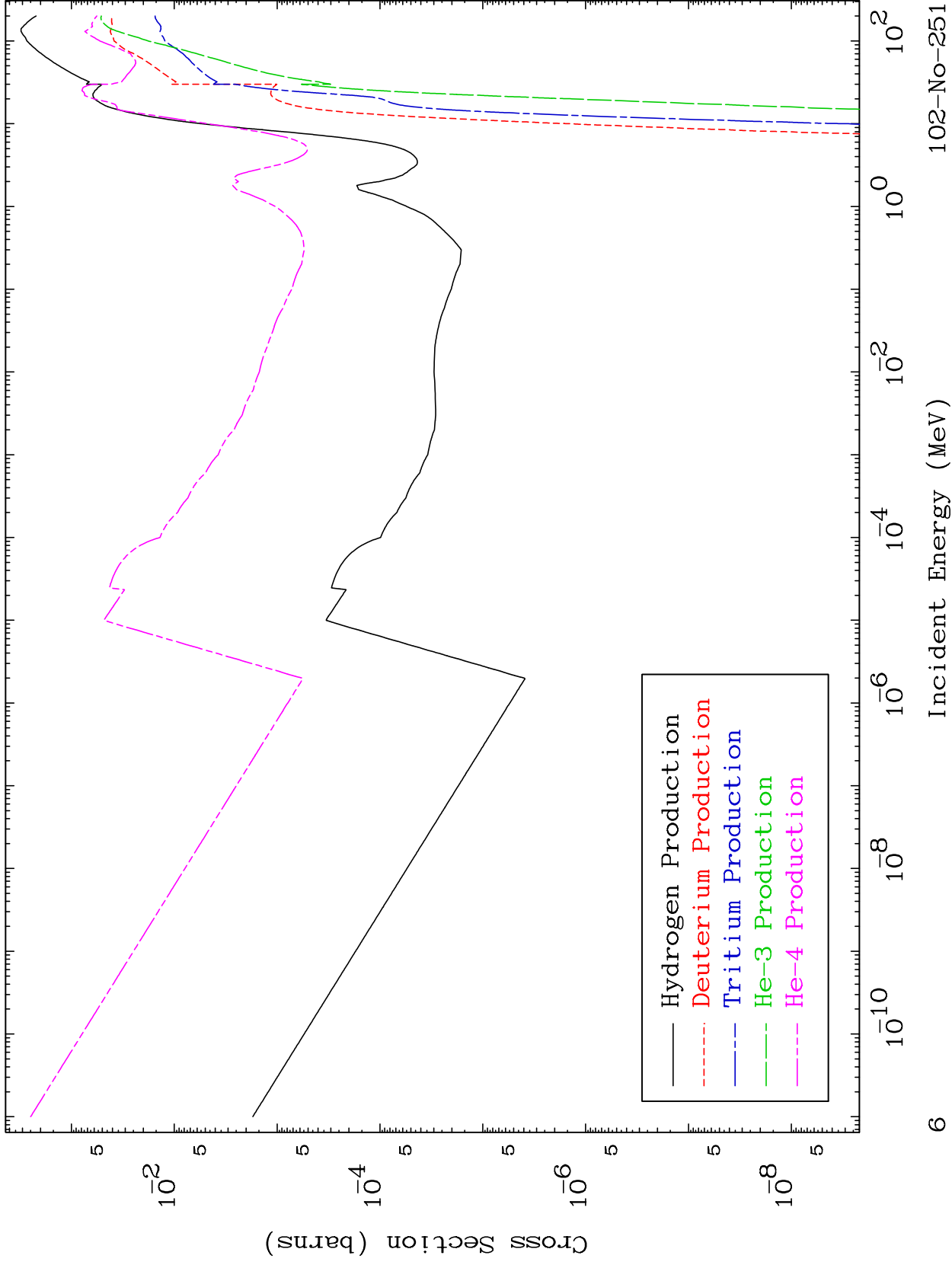
102-No-251



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Particle Production
293 Kelvin Cross Sections

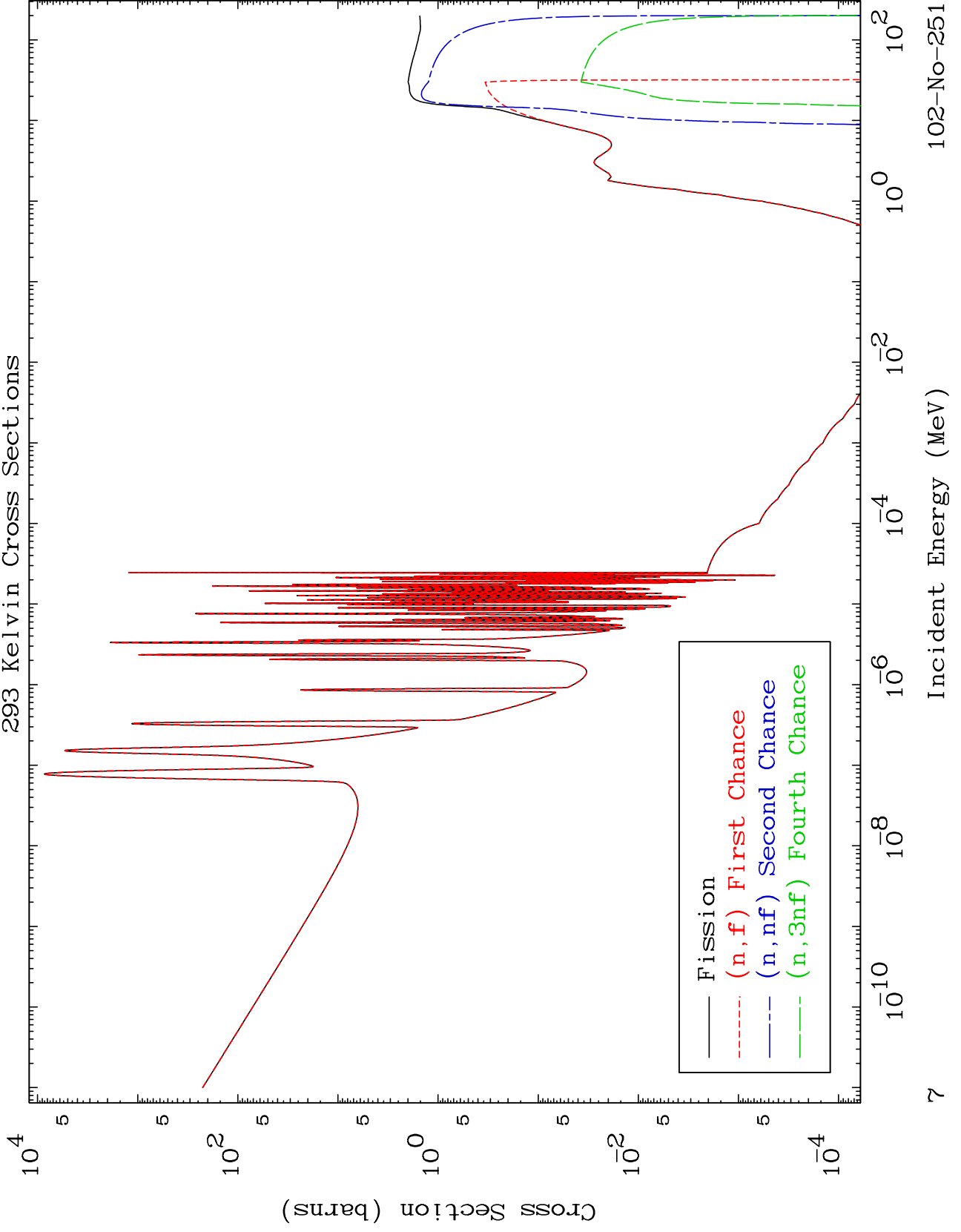
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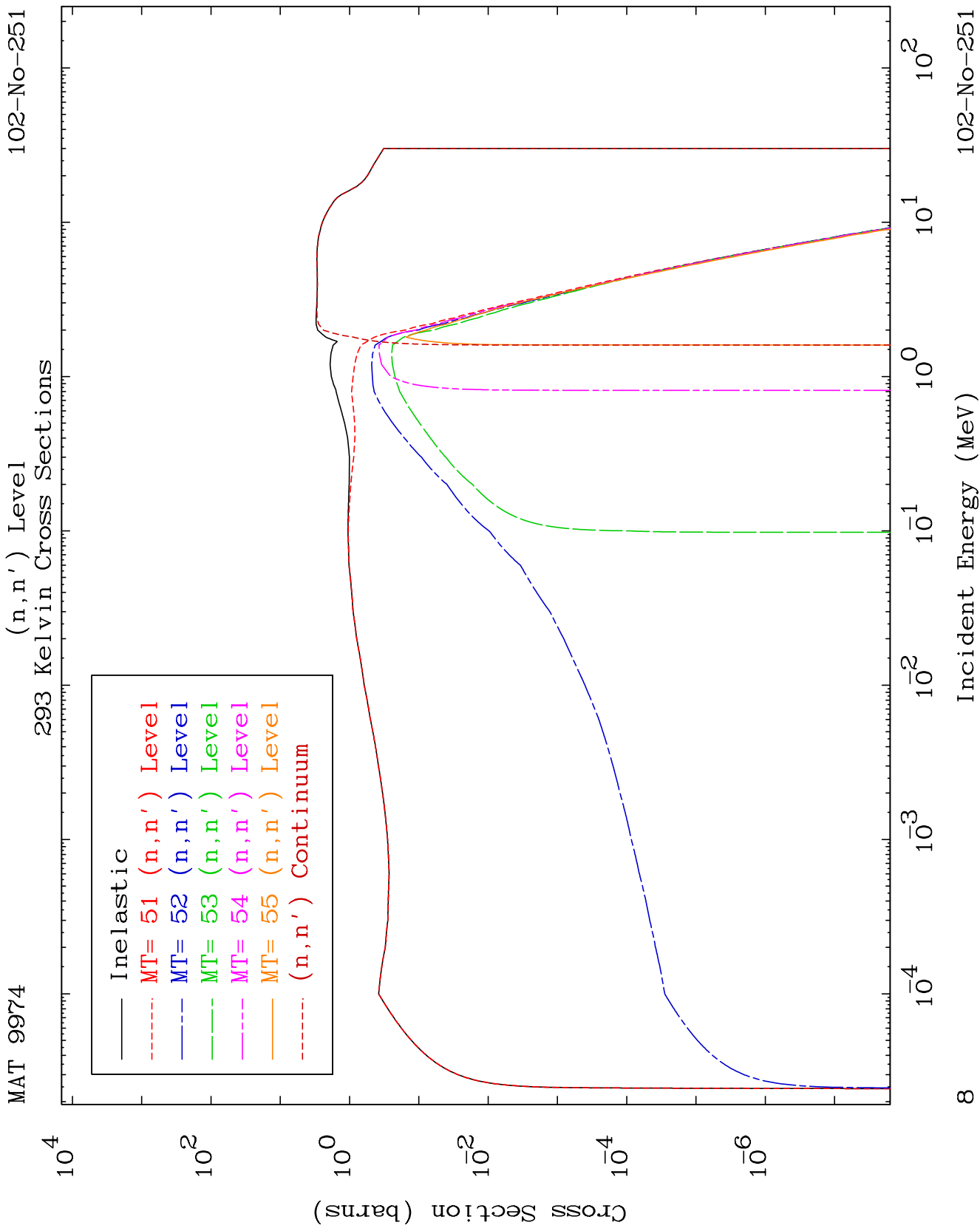


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Fission
293 Kelvin Cross Sections

102-No-251

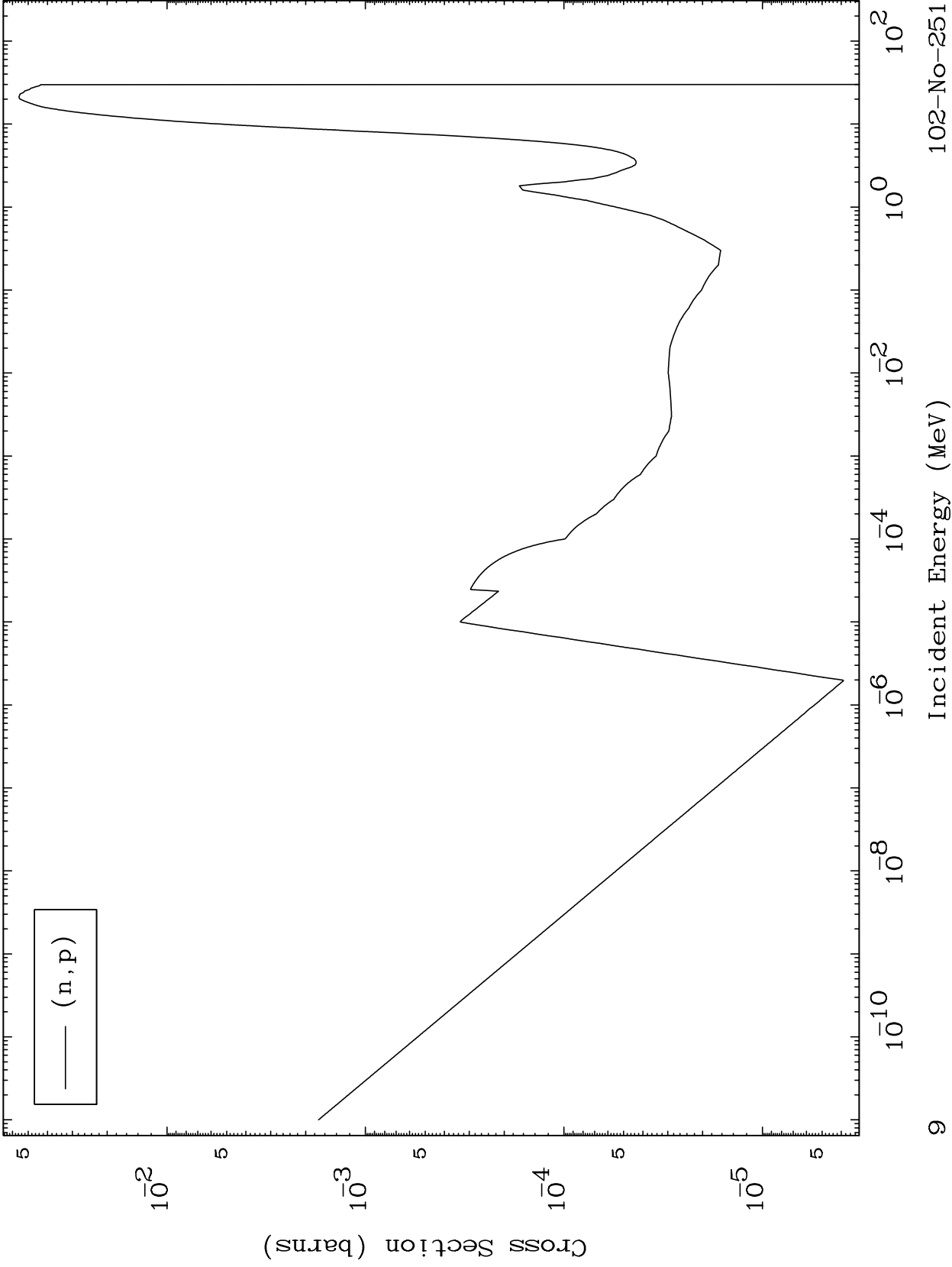




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(n,p) Levels
293 Kelvin Cross Sections

102-No-251

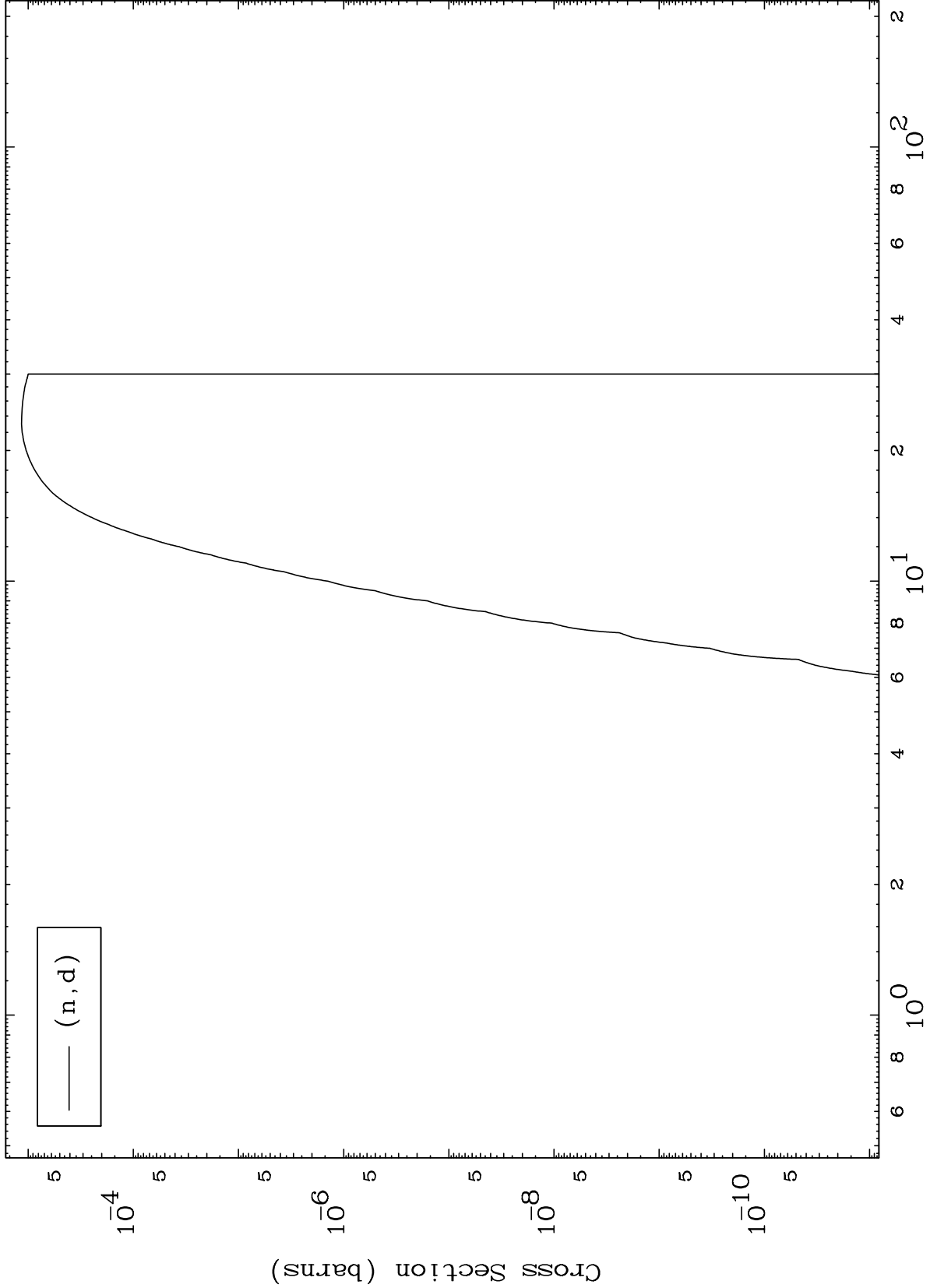


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(n,d) Levels

102-No-251

293 Kelvin Cross Sections



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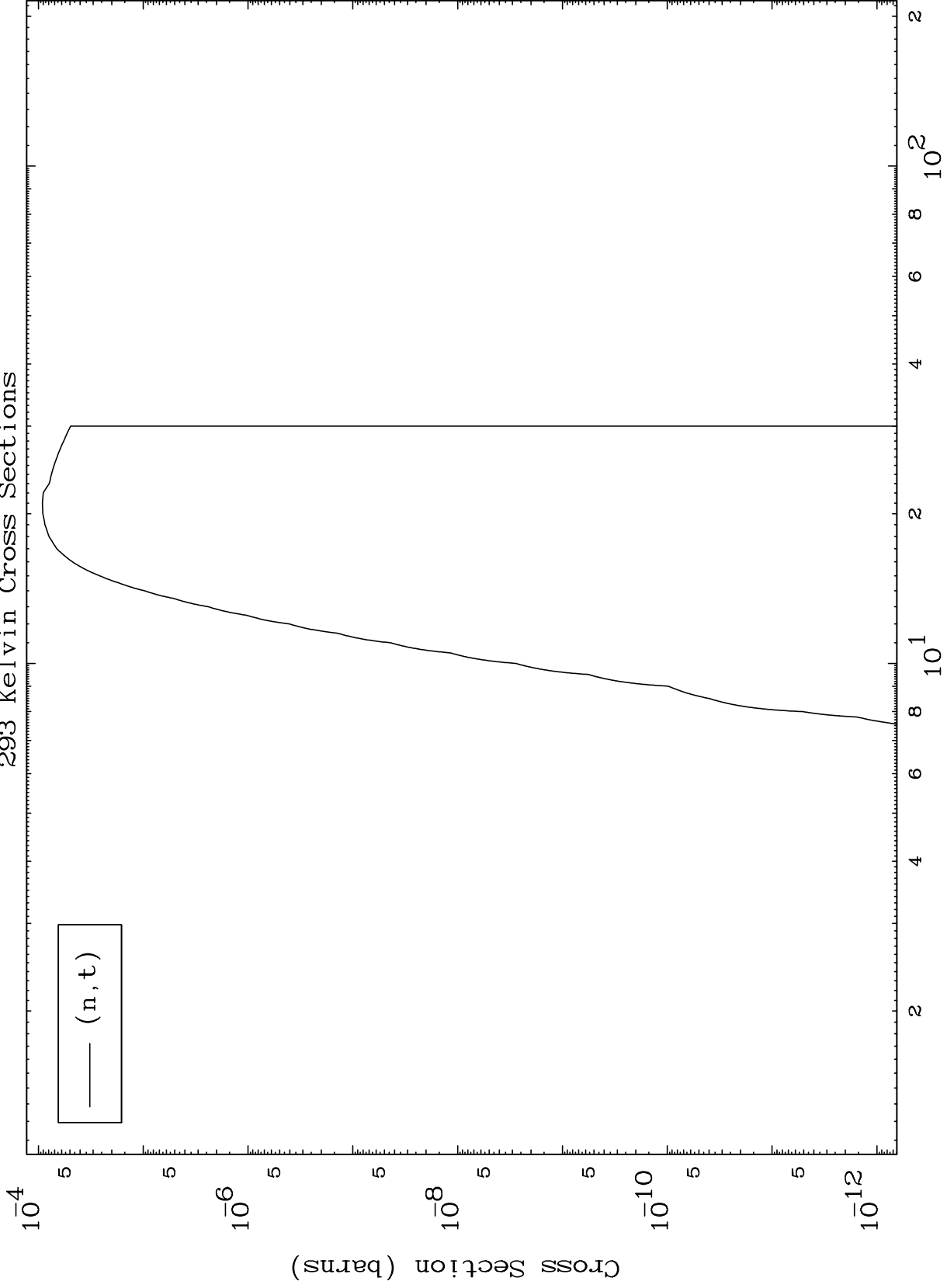
Incident Energy (MeV)

102-No-251

MAT 9974

(n,t) Levels
293 Kelvin Cross Sections

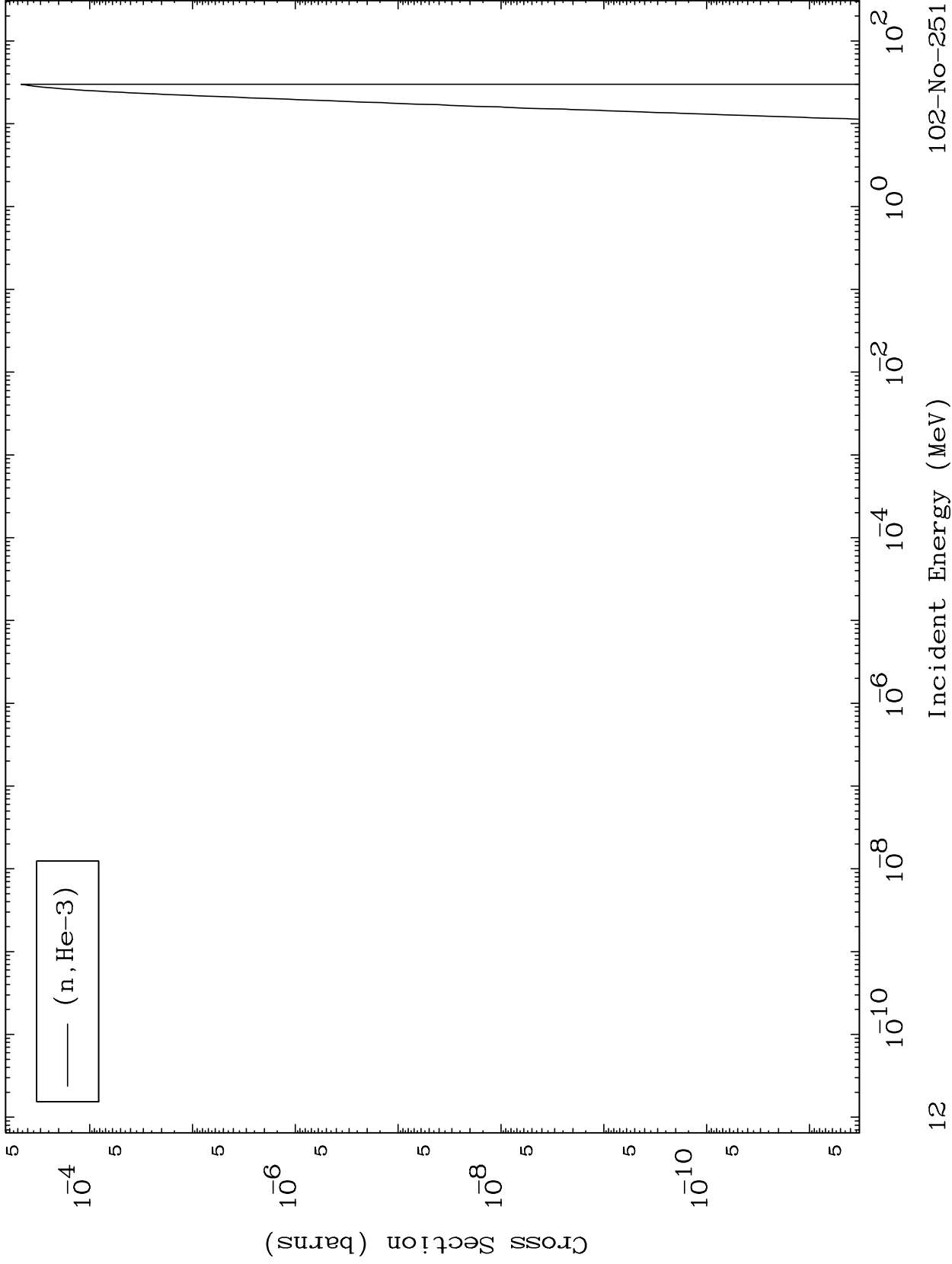
102-No-251



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(n,He3) Levels
293 Kelvin Cross Sections

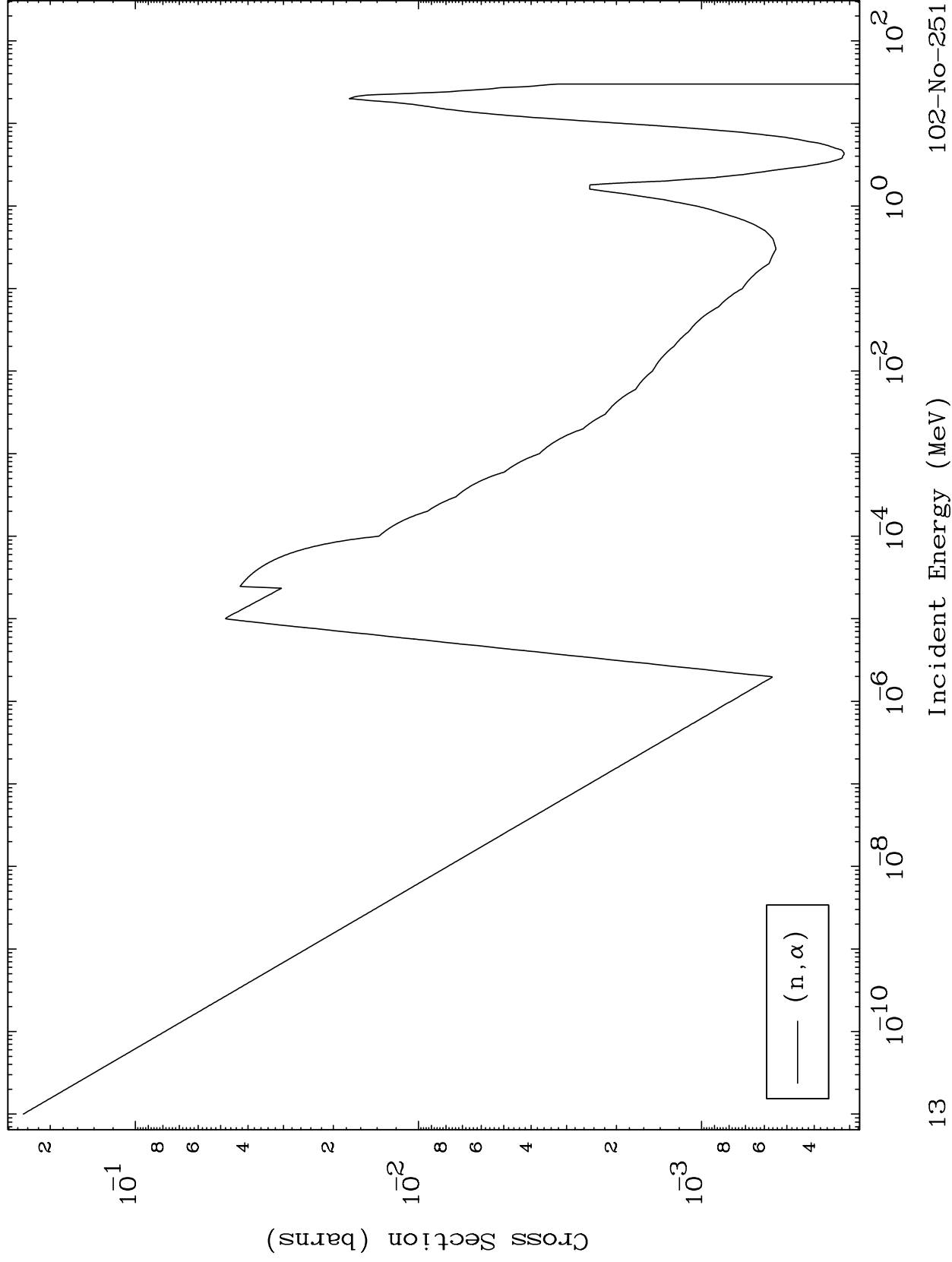
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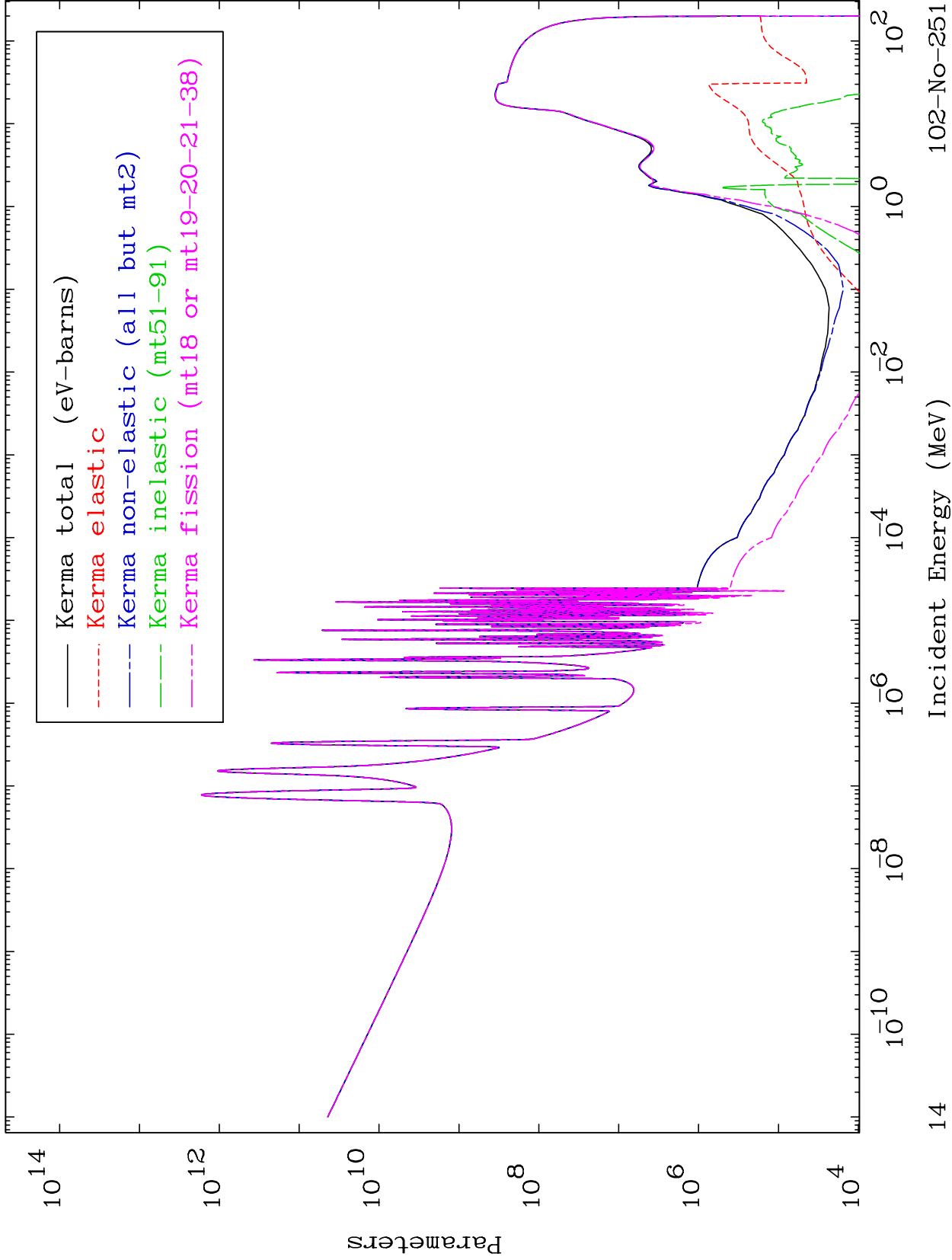


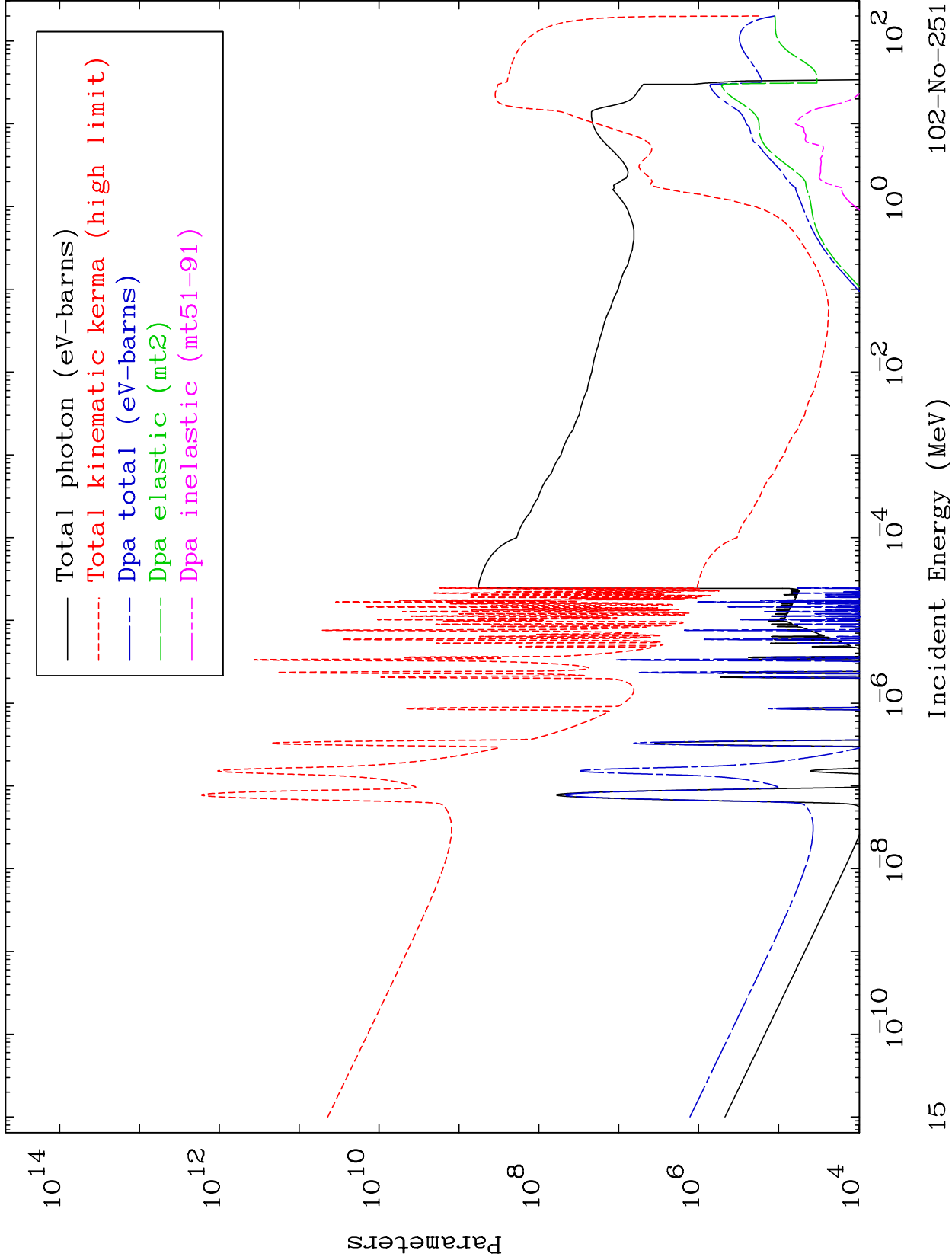
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(n,α) Levels
293 Kelvin Cross Sections

102-No-251



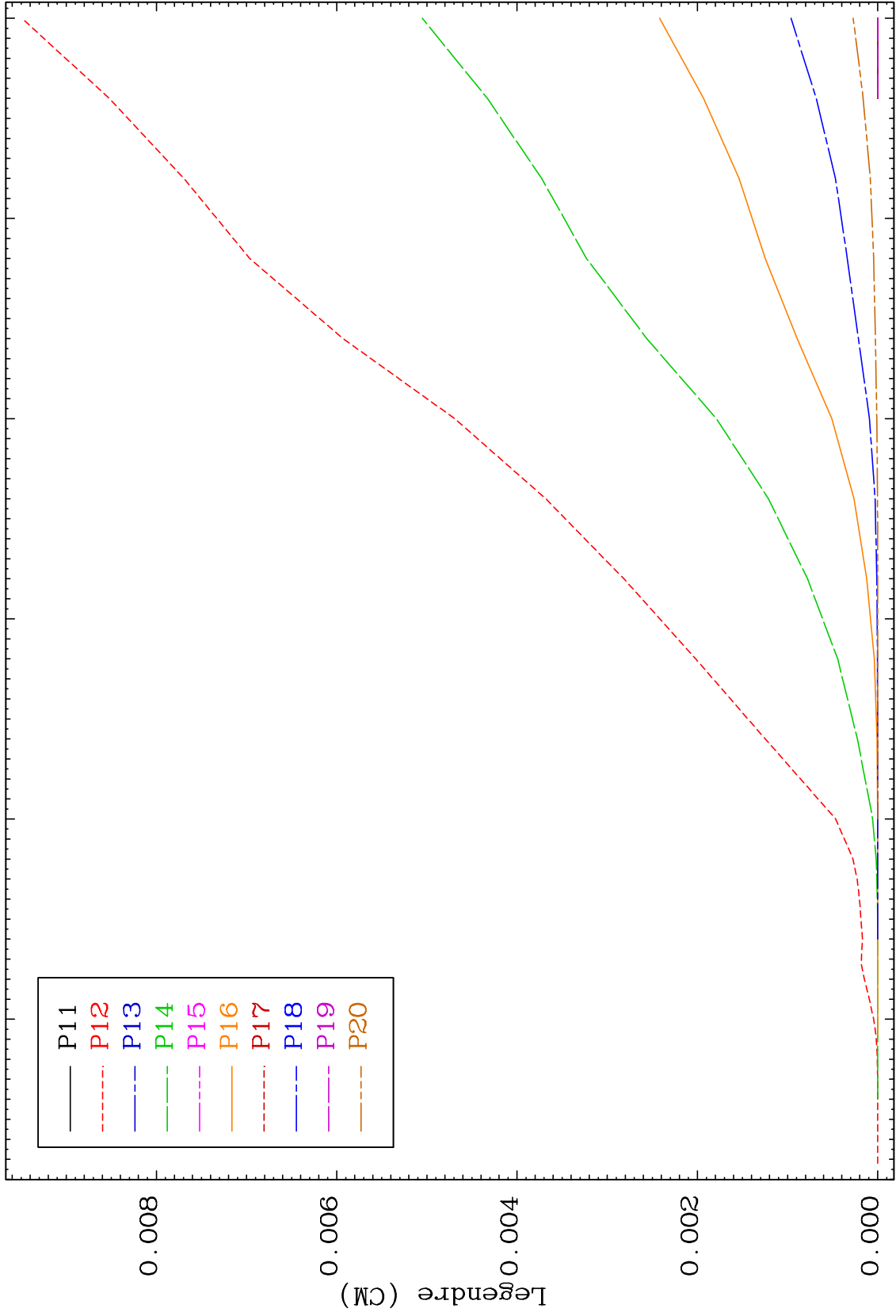




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Elastic Legendre Coefficients

102-No-251



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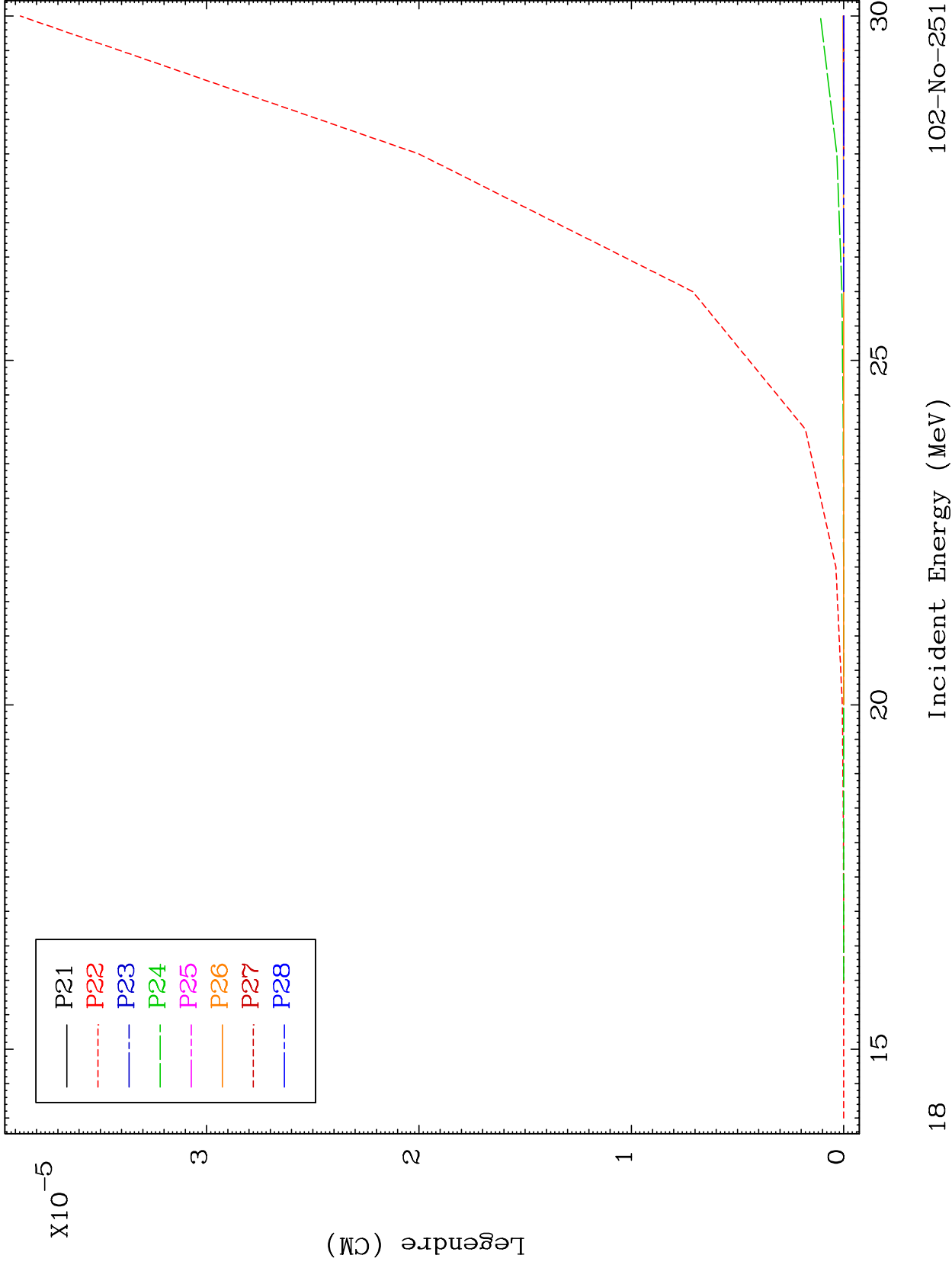
Incident Energy (MeV)

102-No-251

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Elastic
Legendre Coefficients

102-No-251

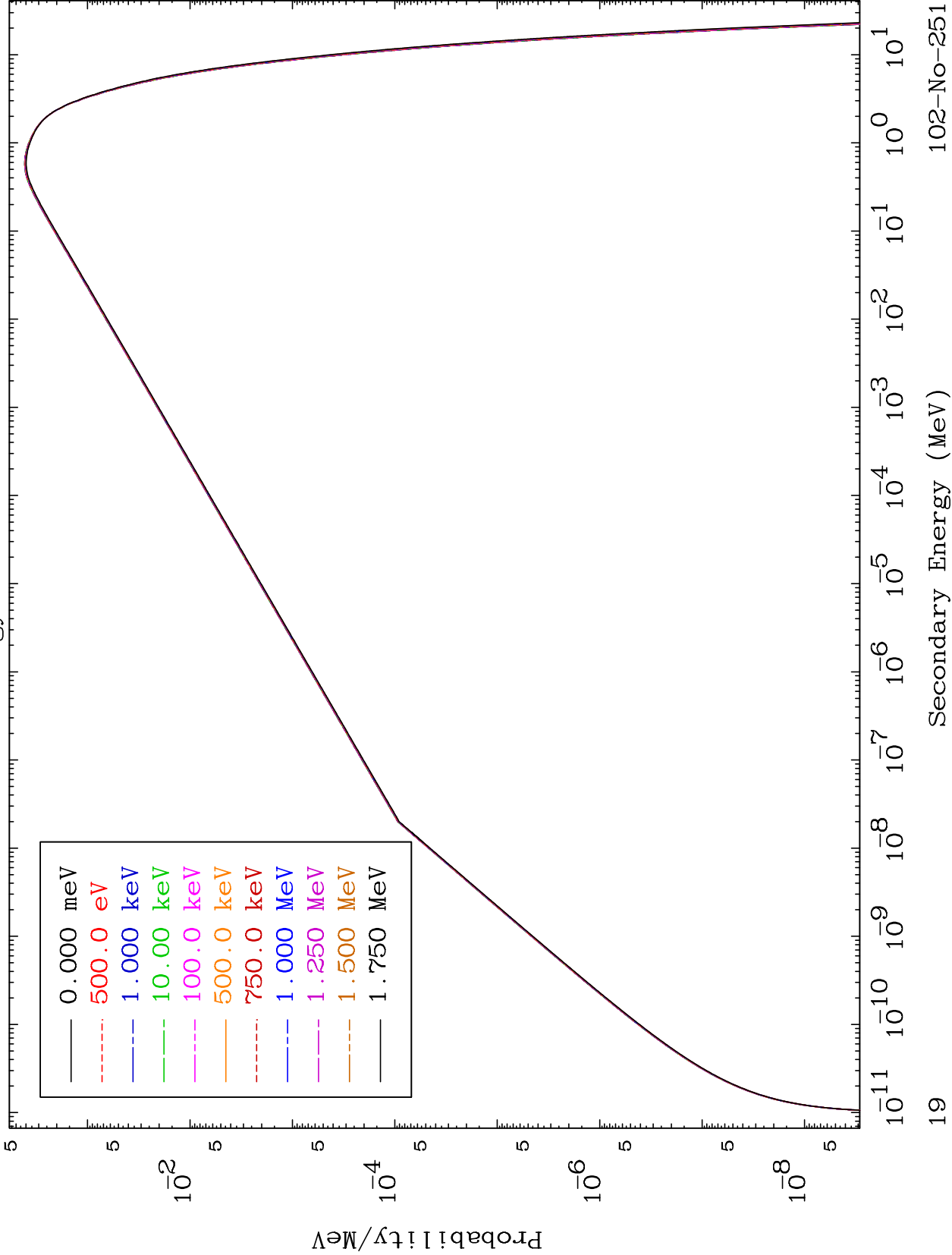


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Incident Energy (MeV)

102-No-251

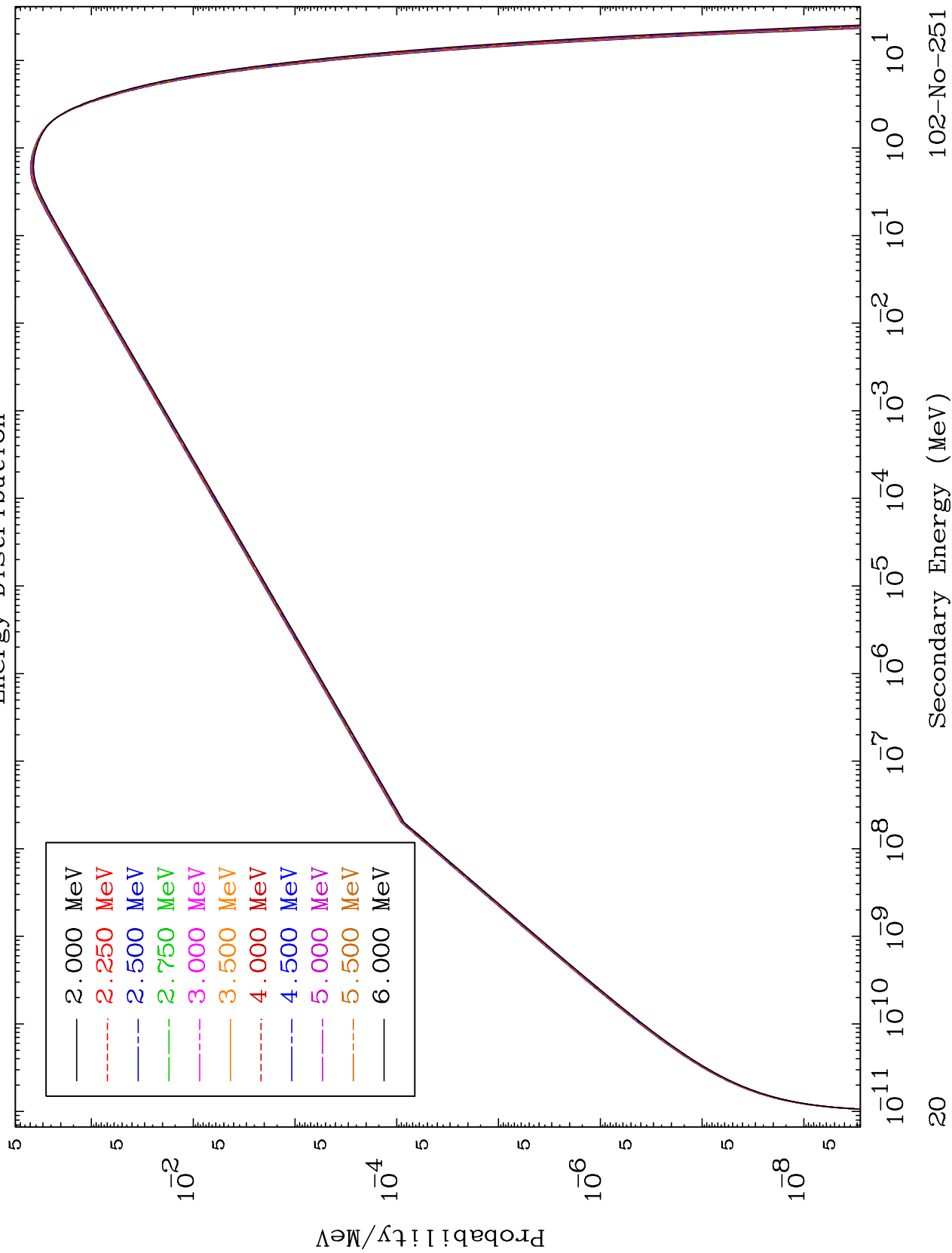
Fission Energy Distribution



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Fission Energy Distribution

102-No-251



102-No-251

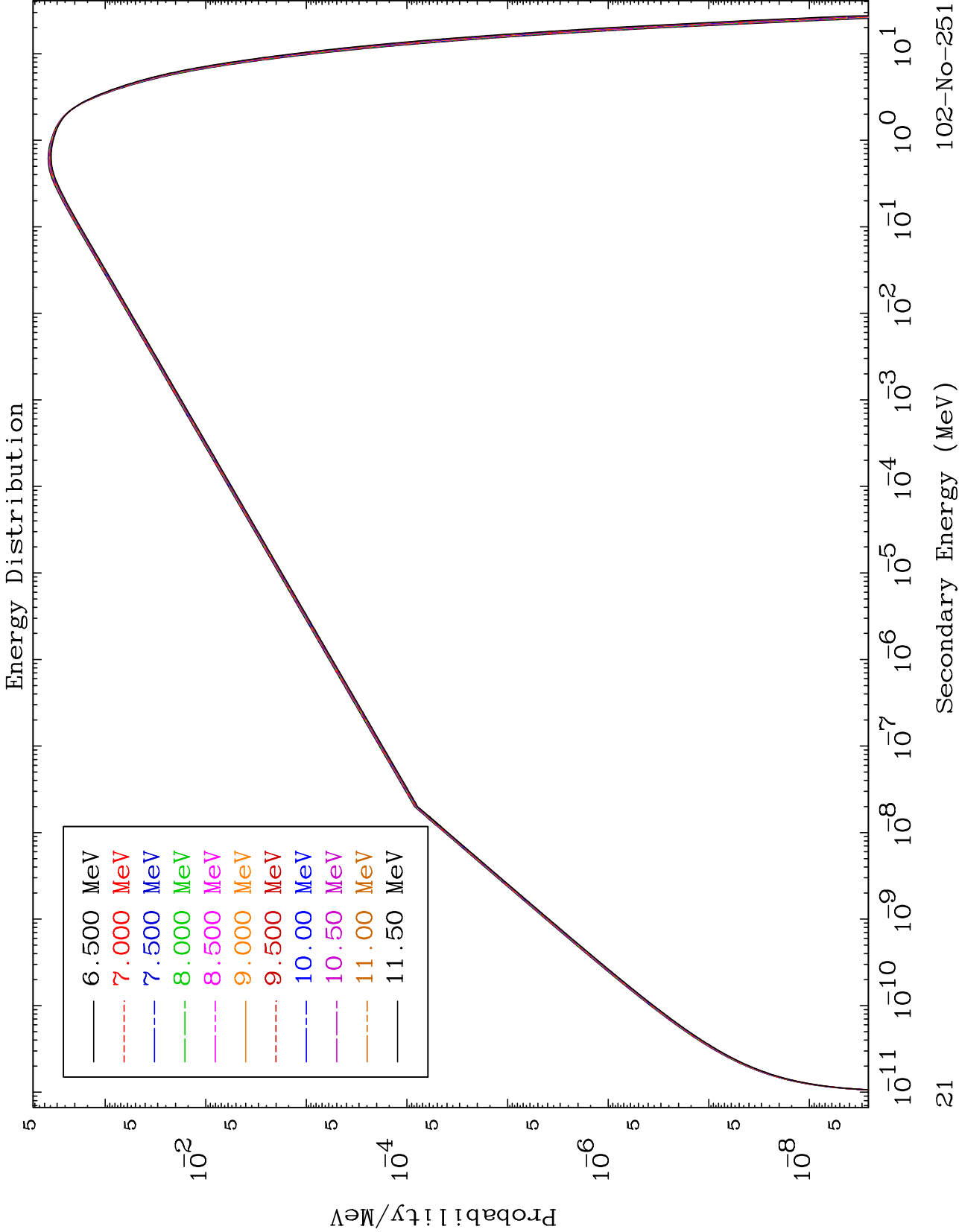
Secondary Energy (MeV)

20

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Fission

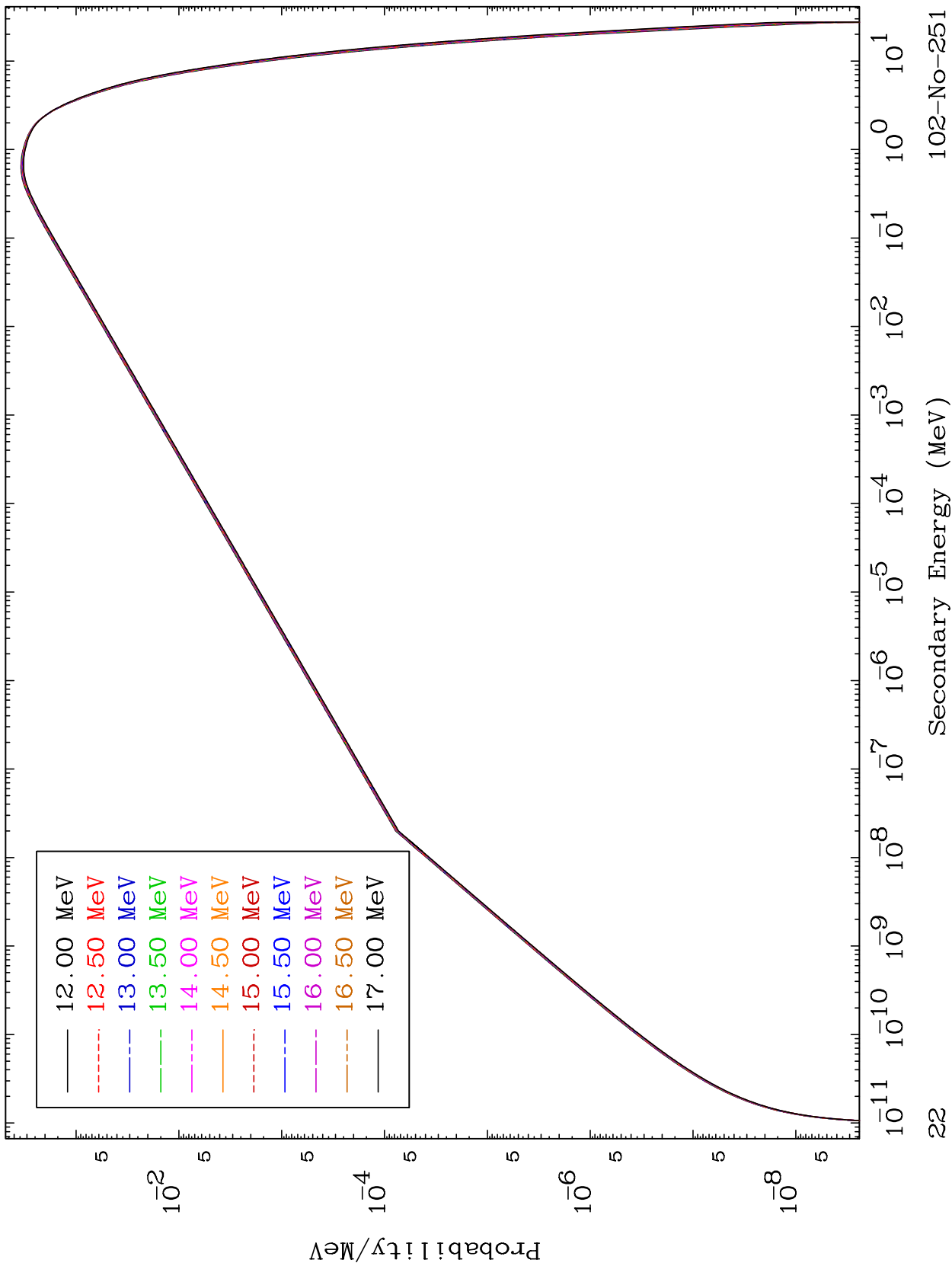
102-No-251



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Fission Energy Distribution

102-No-251

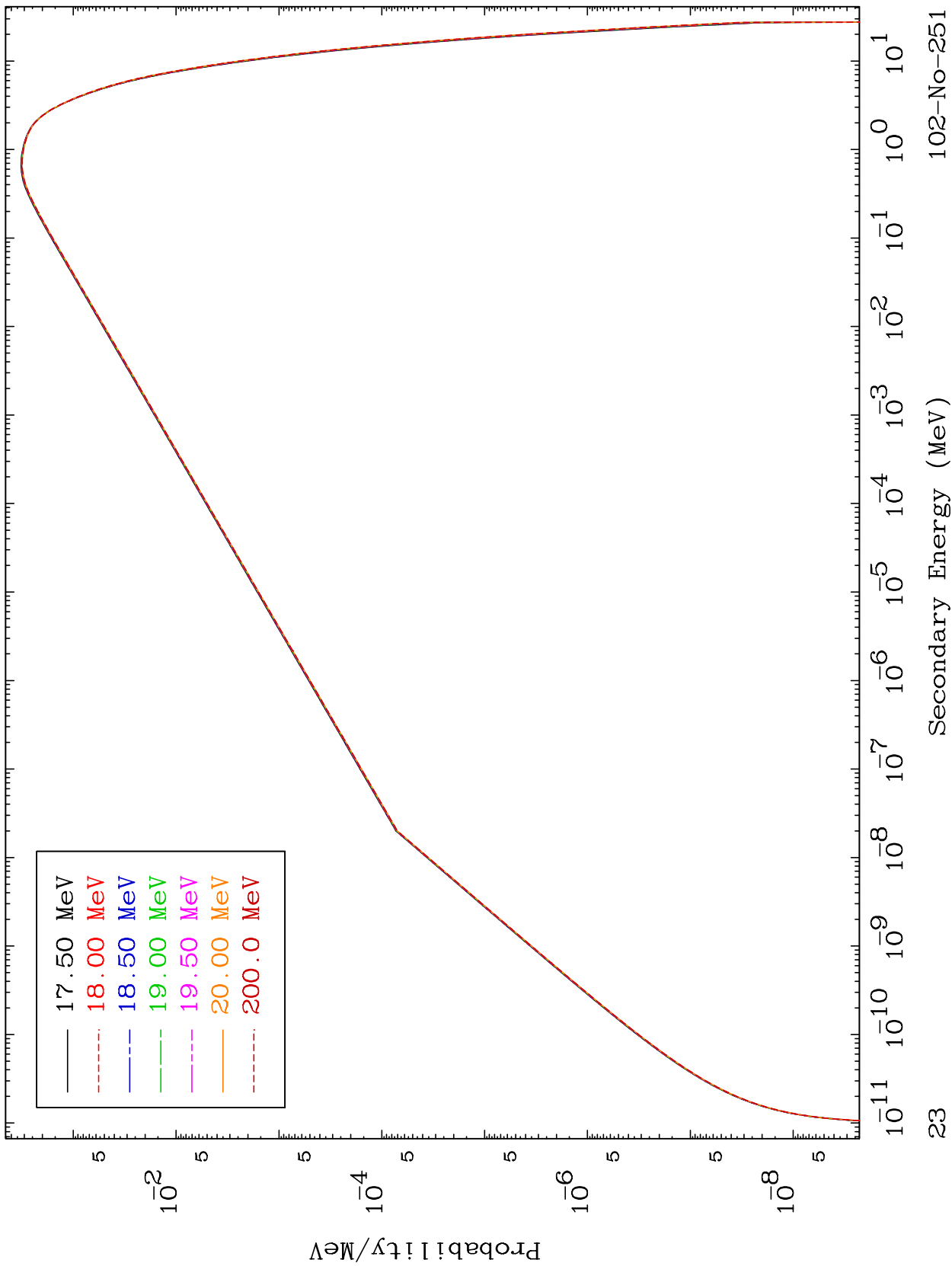


102-No-251

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Fission Energy Distribution

102-No-251



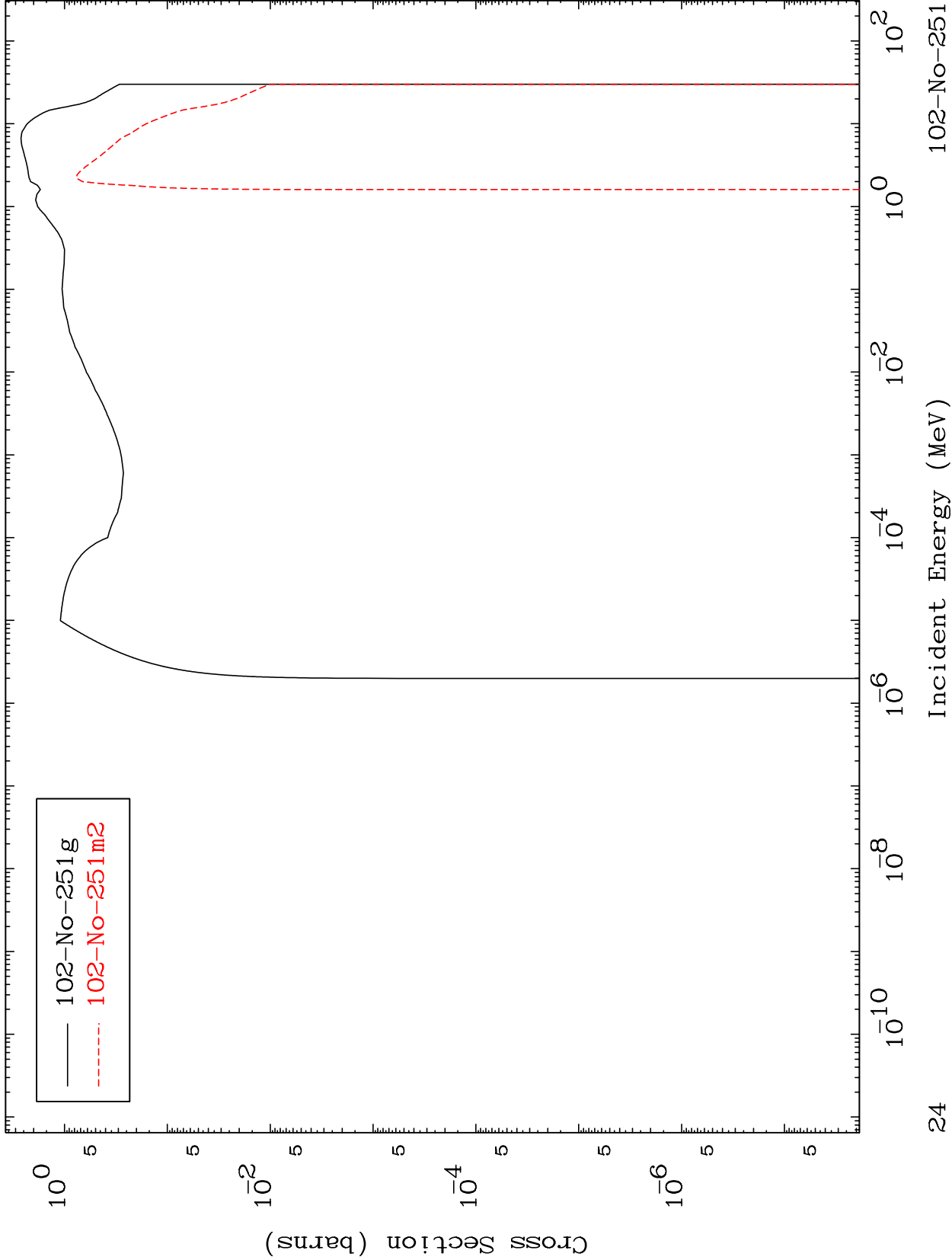
102-No-251

Secondary Energy (MeV)

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Inelastic
Radionuclide Production Cross Section

102-No-251

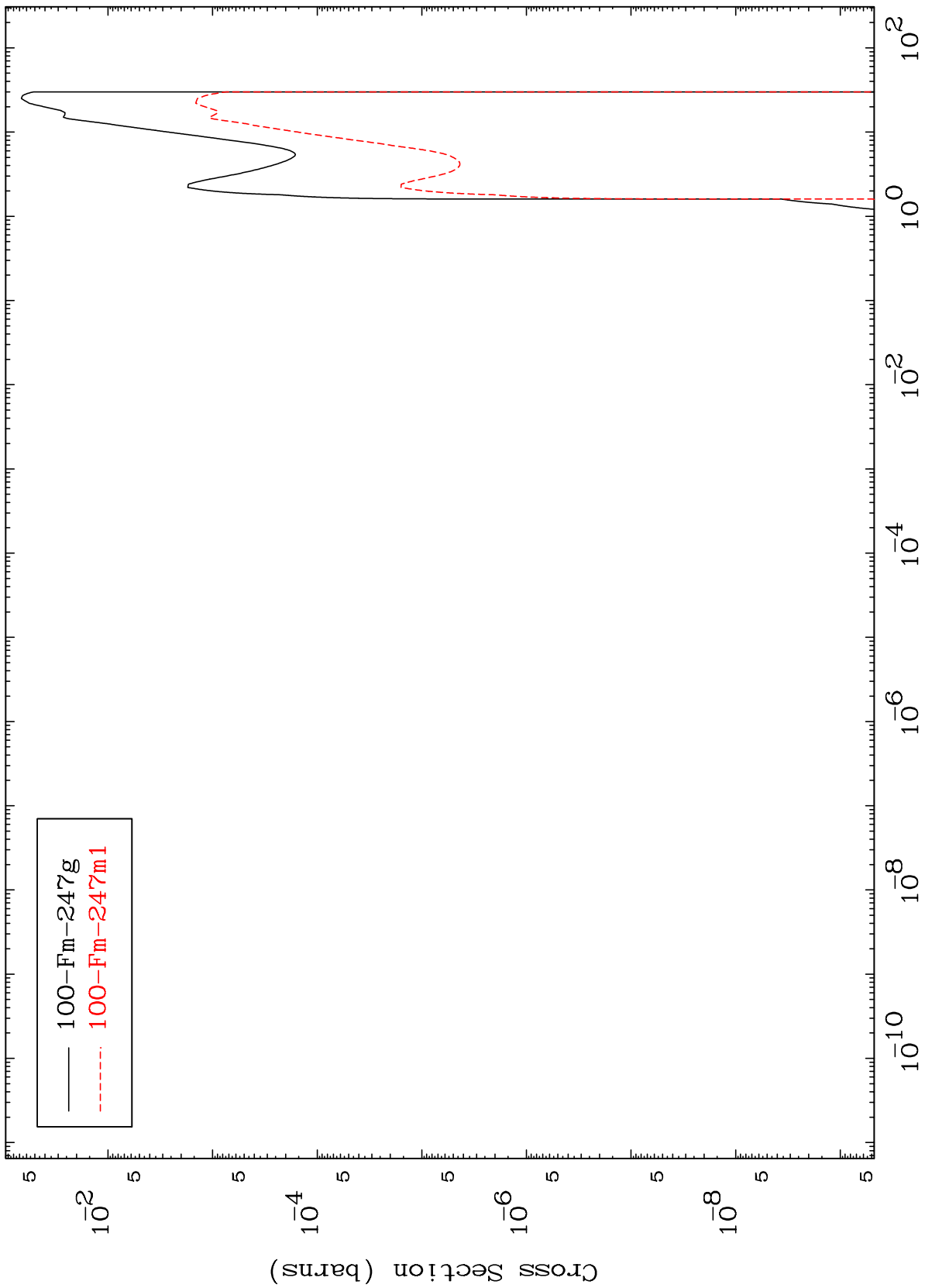


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$(n, n') \alpha$

102-No-251

Radionuclide Production Cross Section

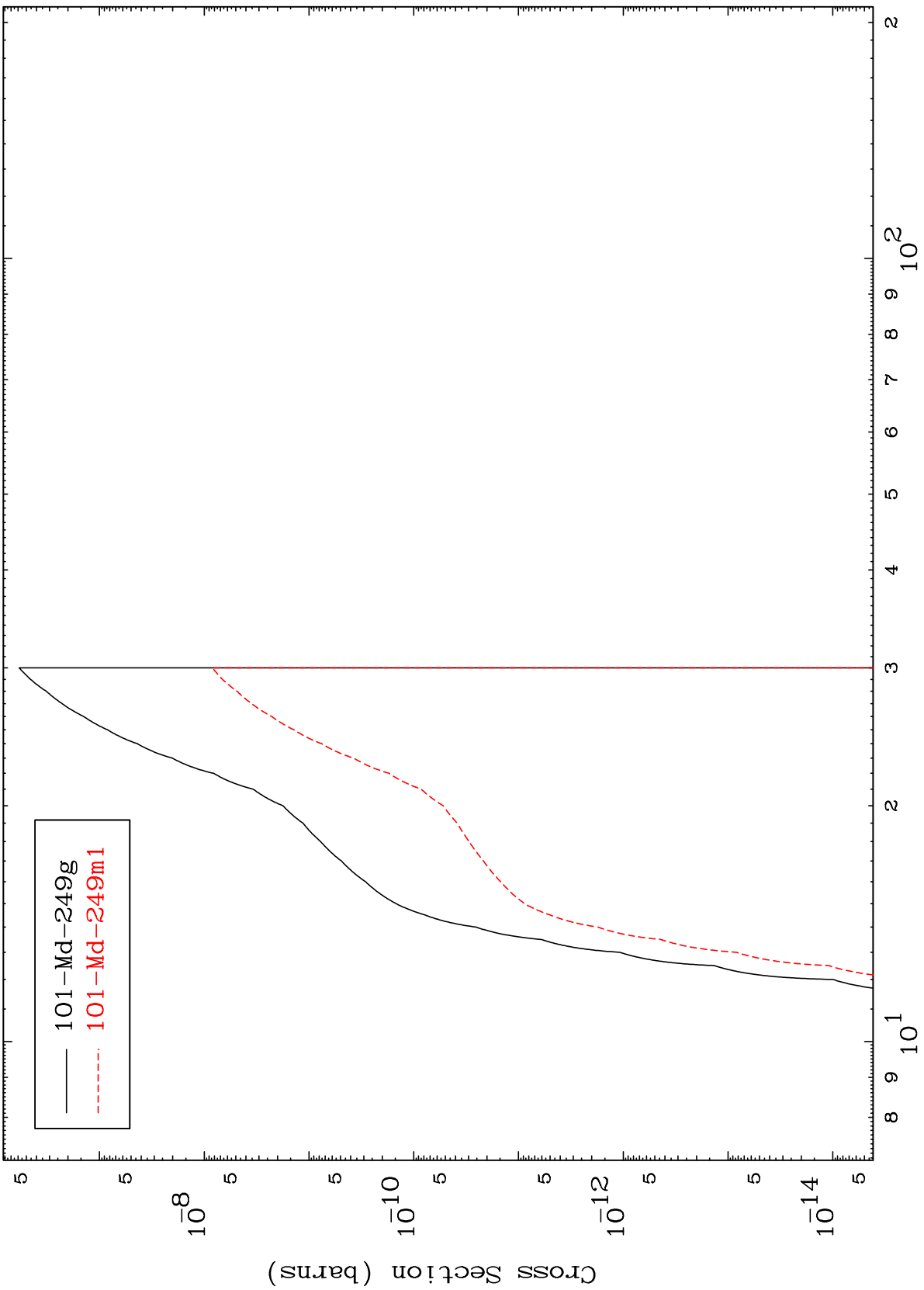


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(n,n') d

102-No-251

Radionuclide Production Cross Section



26

Incident Energy (MeV)

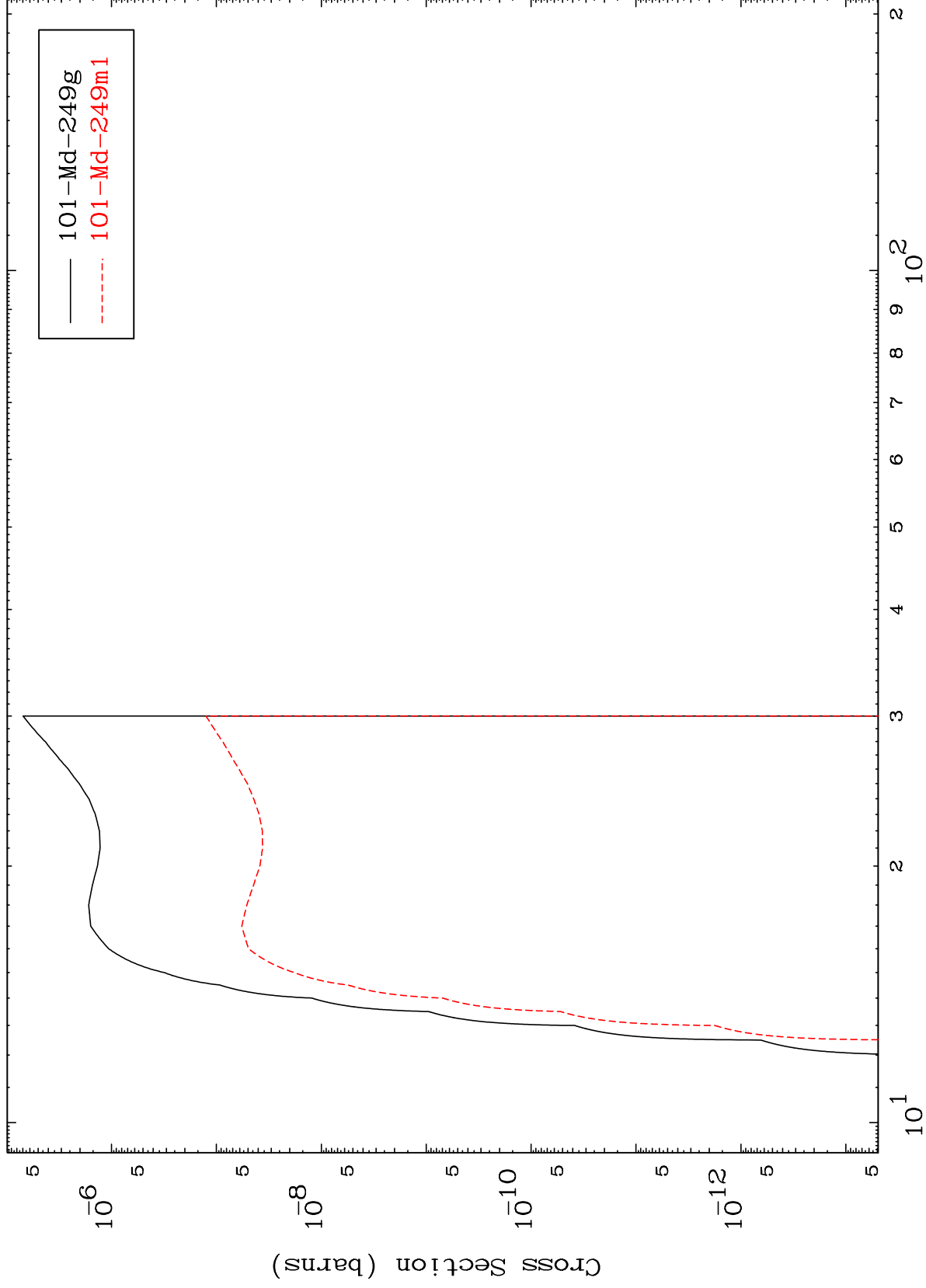
102-No-251

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(n,2n) p

102-No-251

Radionuclide Production Cross Section



27

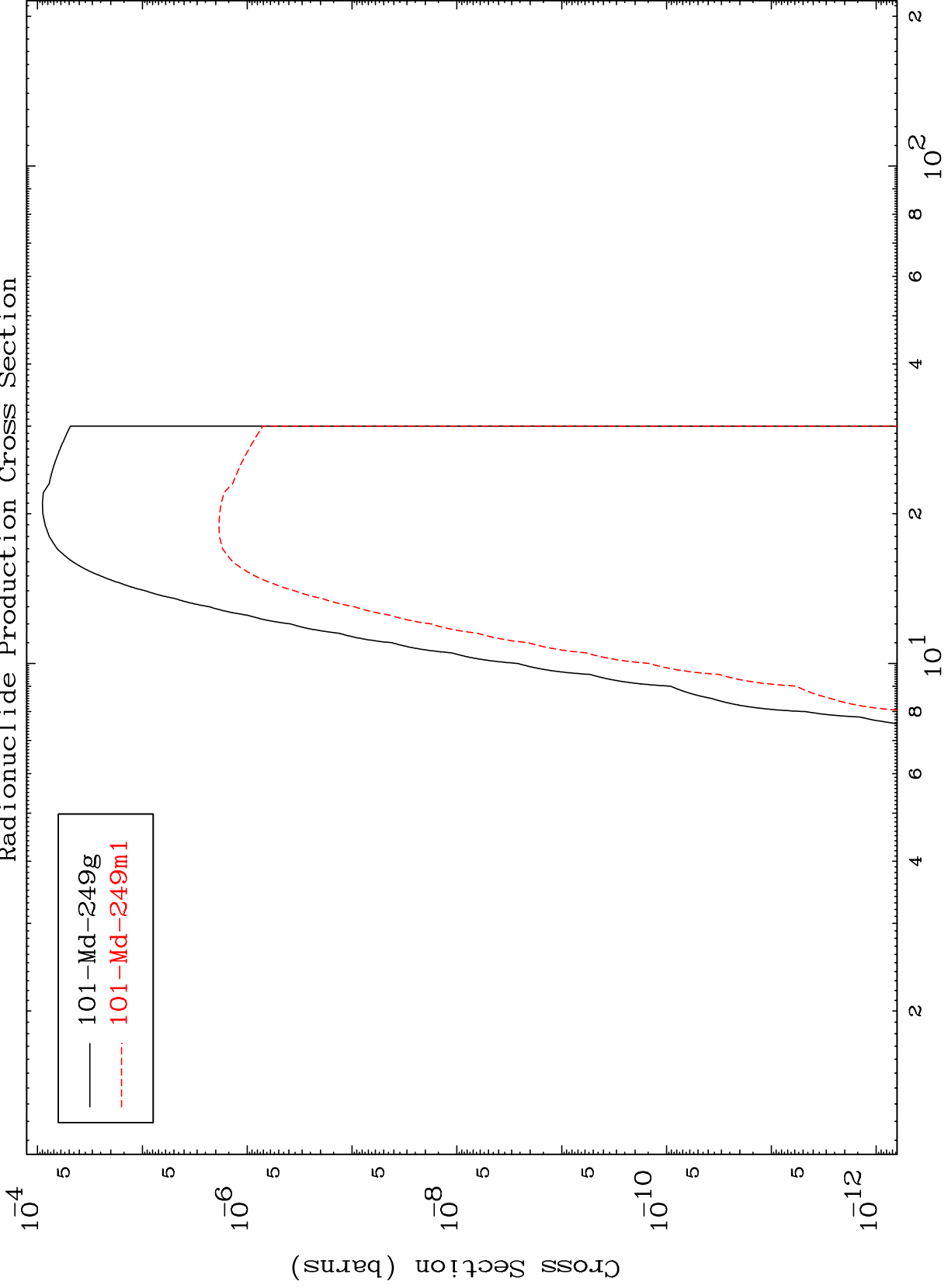
Incident Energy (MeV)

102-No-251

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102-No-251

(n, t)
Radionuclide Production Cross Section



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102-No-251

Incident Energy (MeV)

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102-No-251

(n,2p)
Radionuclide Production Cross Section

