

Program EVALPLOT
(Version 2018-1)

by

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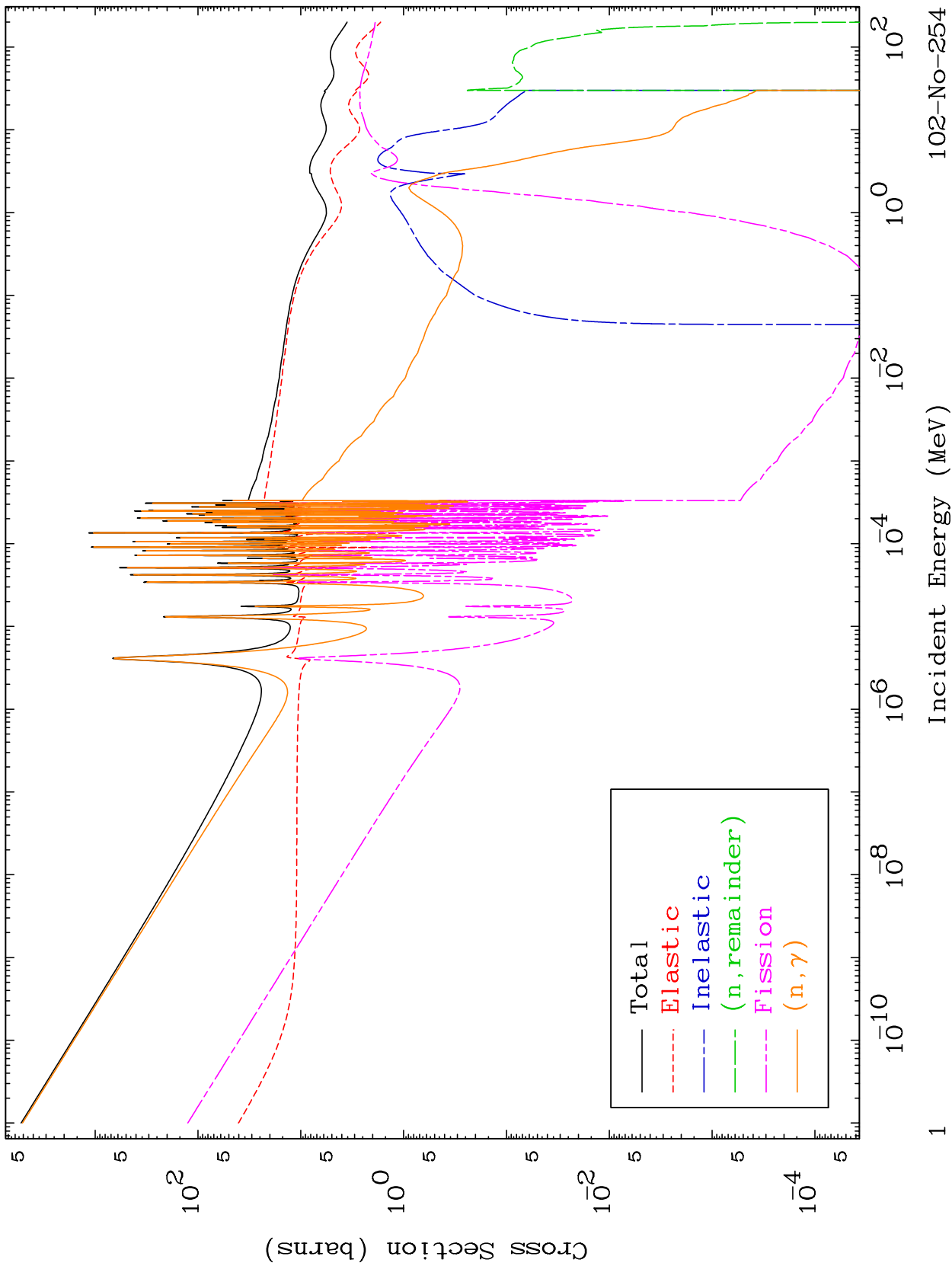
Press Mouse Button to Start

MAT 9972

Major

293 Kelvin Cross Sections

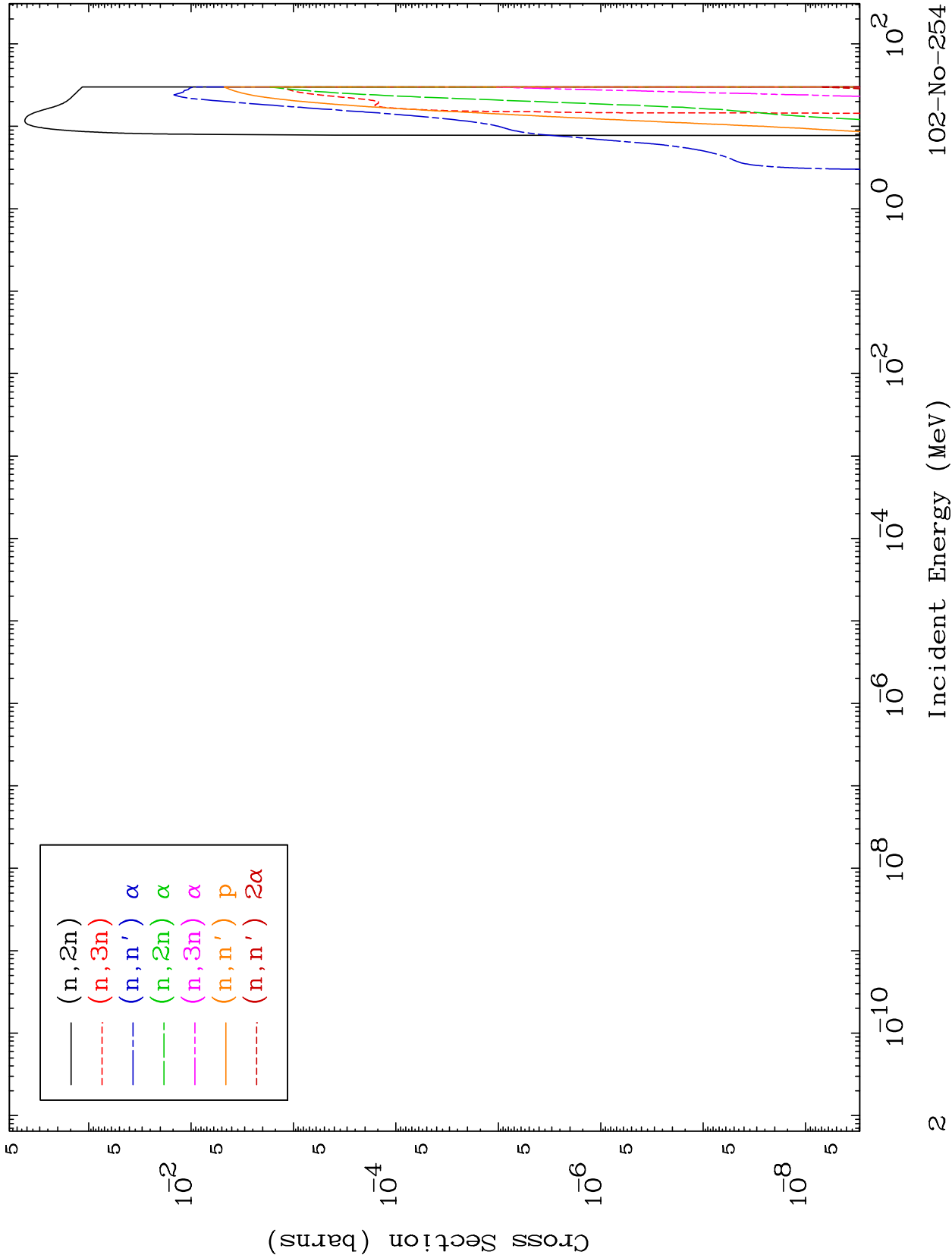
102-No-254

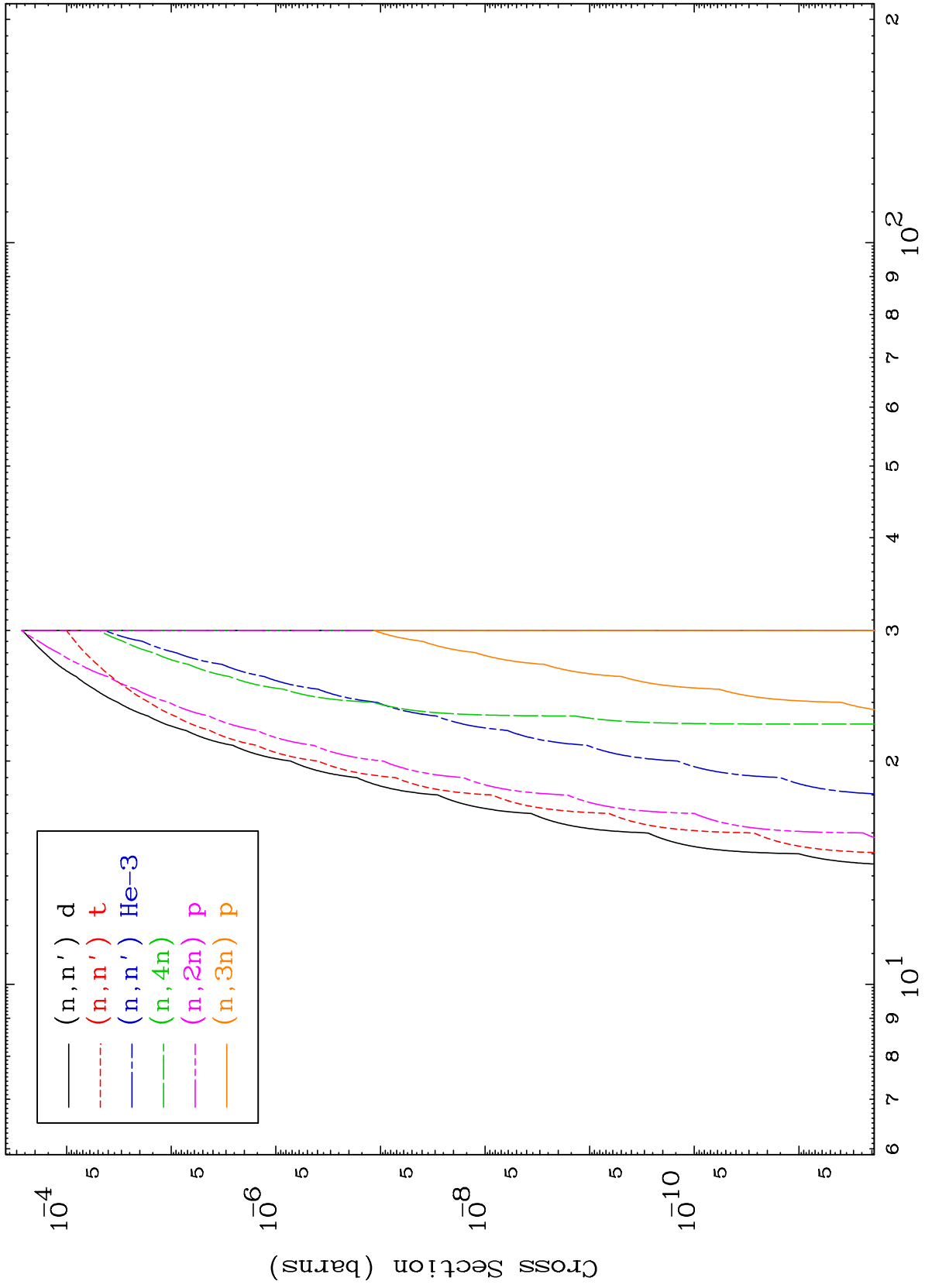


MAT 9972

Neutron Production
293 Kelvin Cross Sections

102-No-254

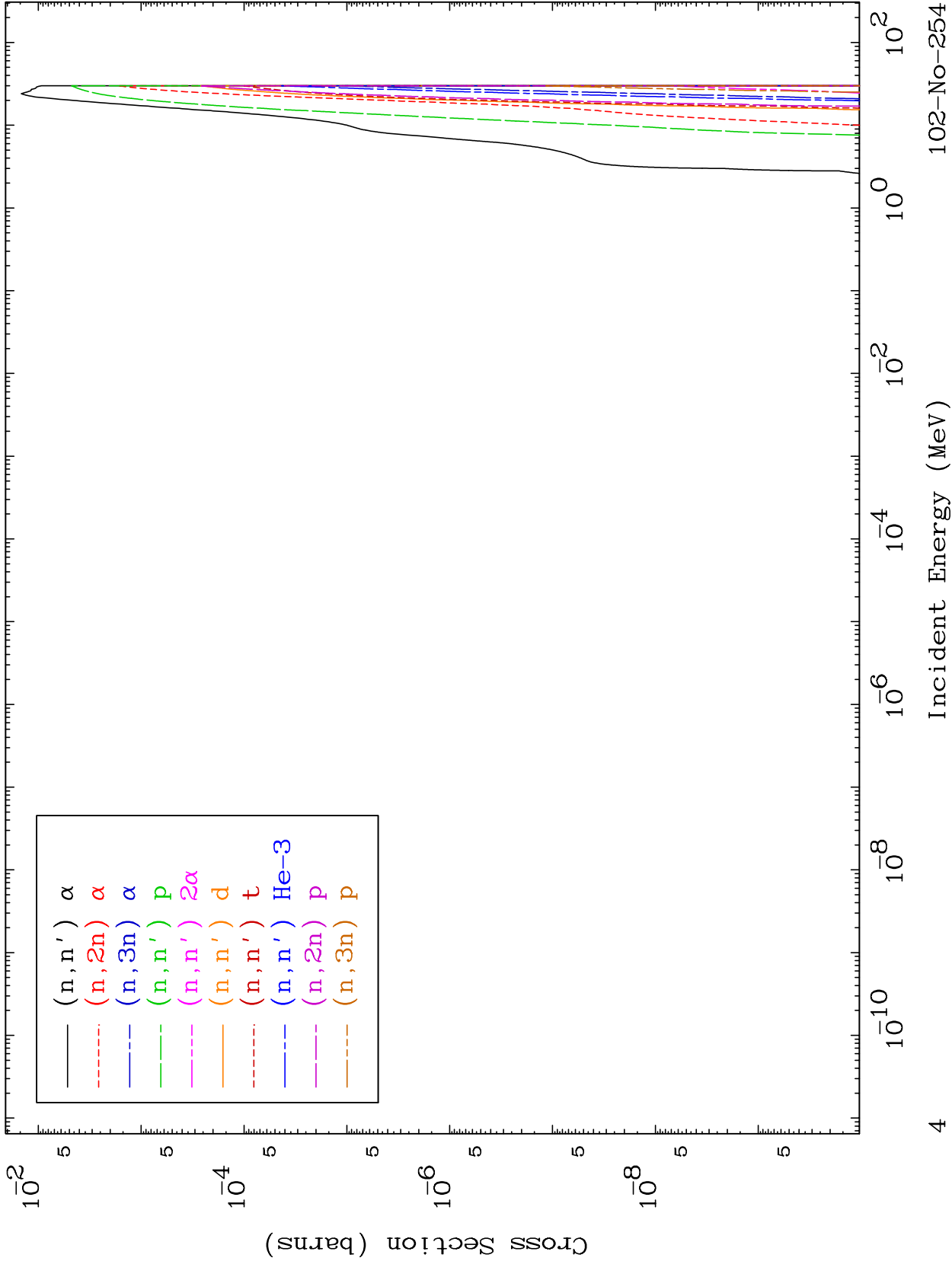




MAT 9972

Charged Particle
293 Kelvin Cross Sections

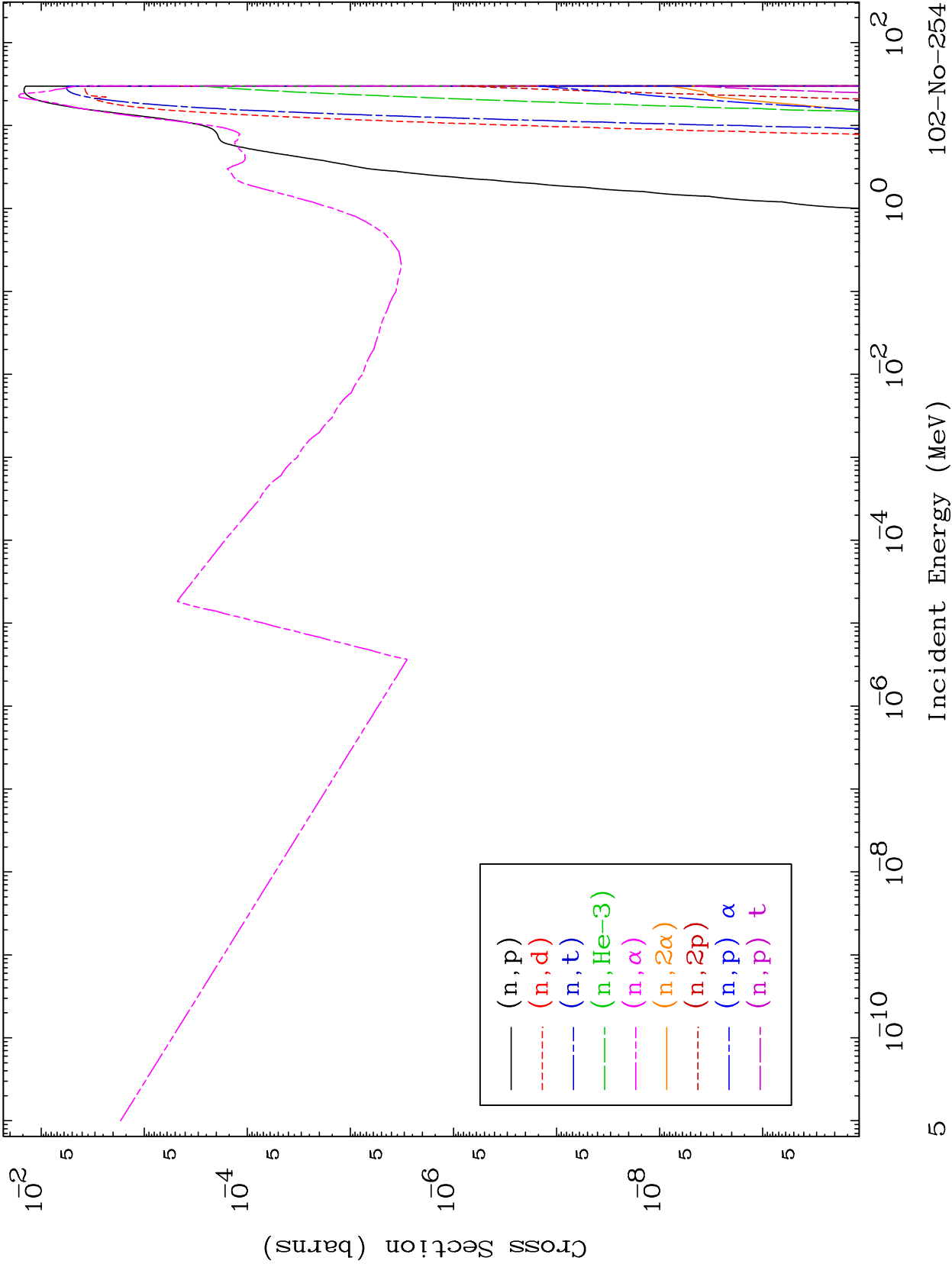
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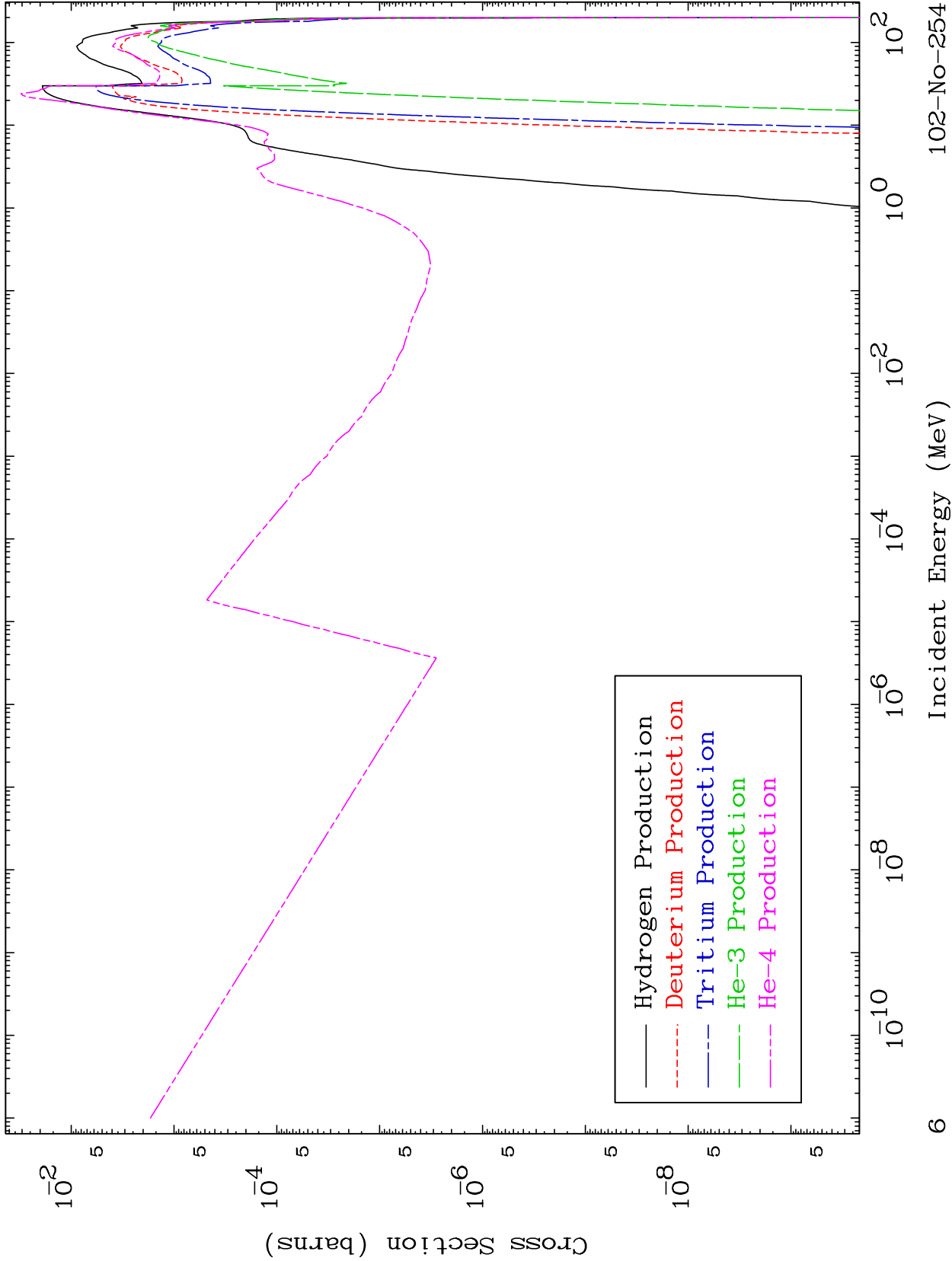


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Charged Particle
293 Kelvin Cross Sections

102-No-254

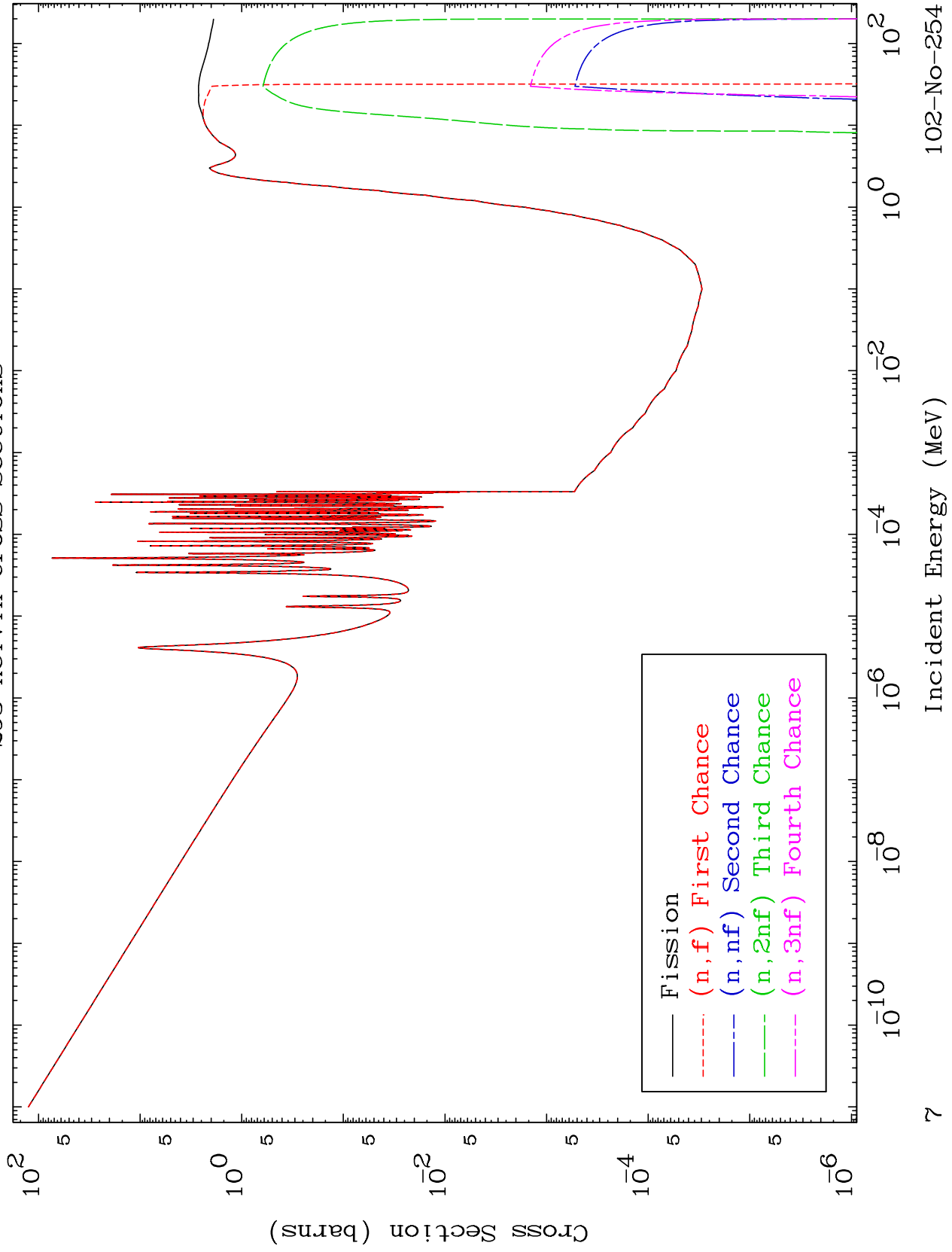




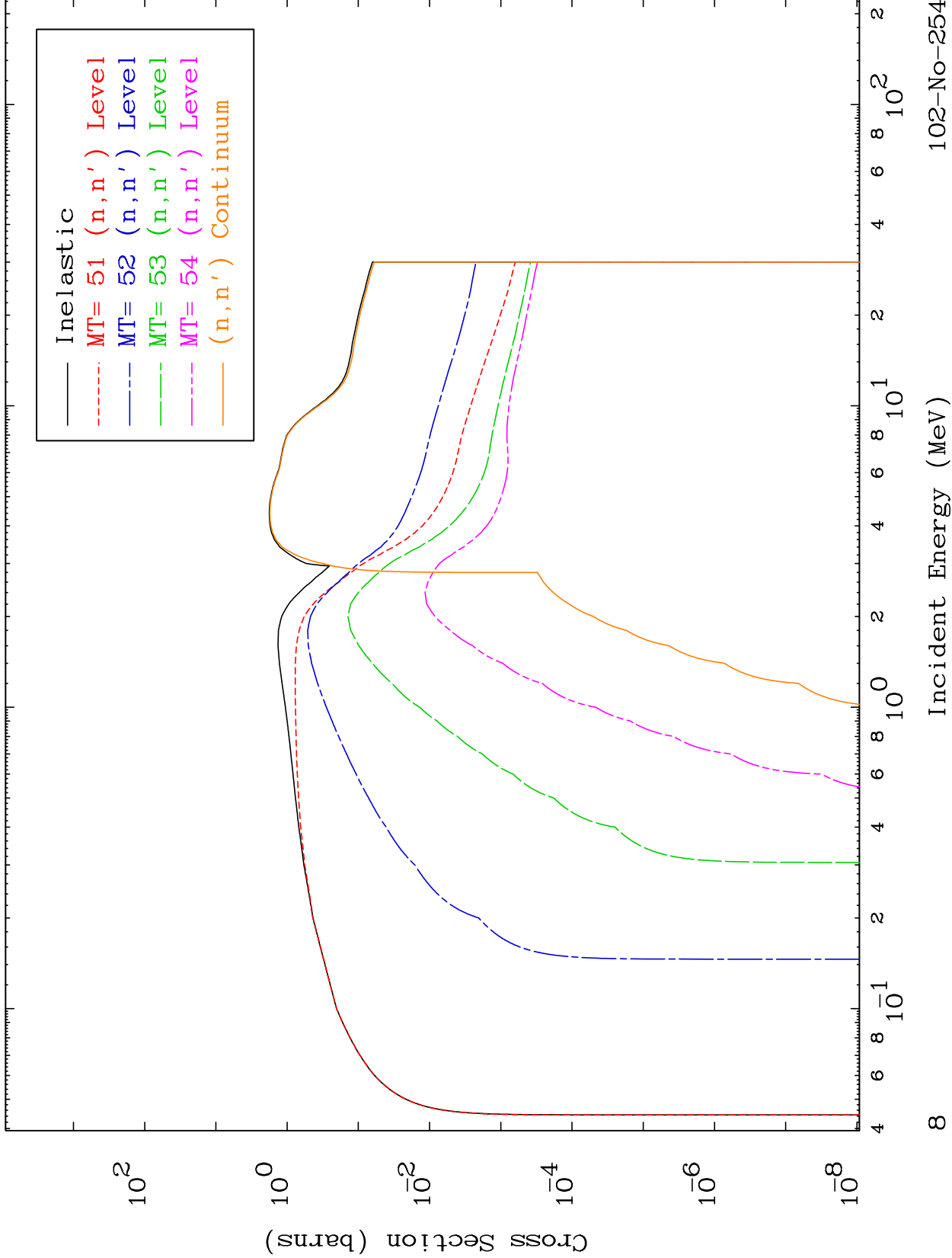
MAT 9972

Fission
293 Kelvin Cross Sections

102-No-254



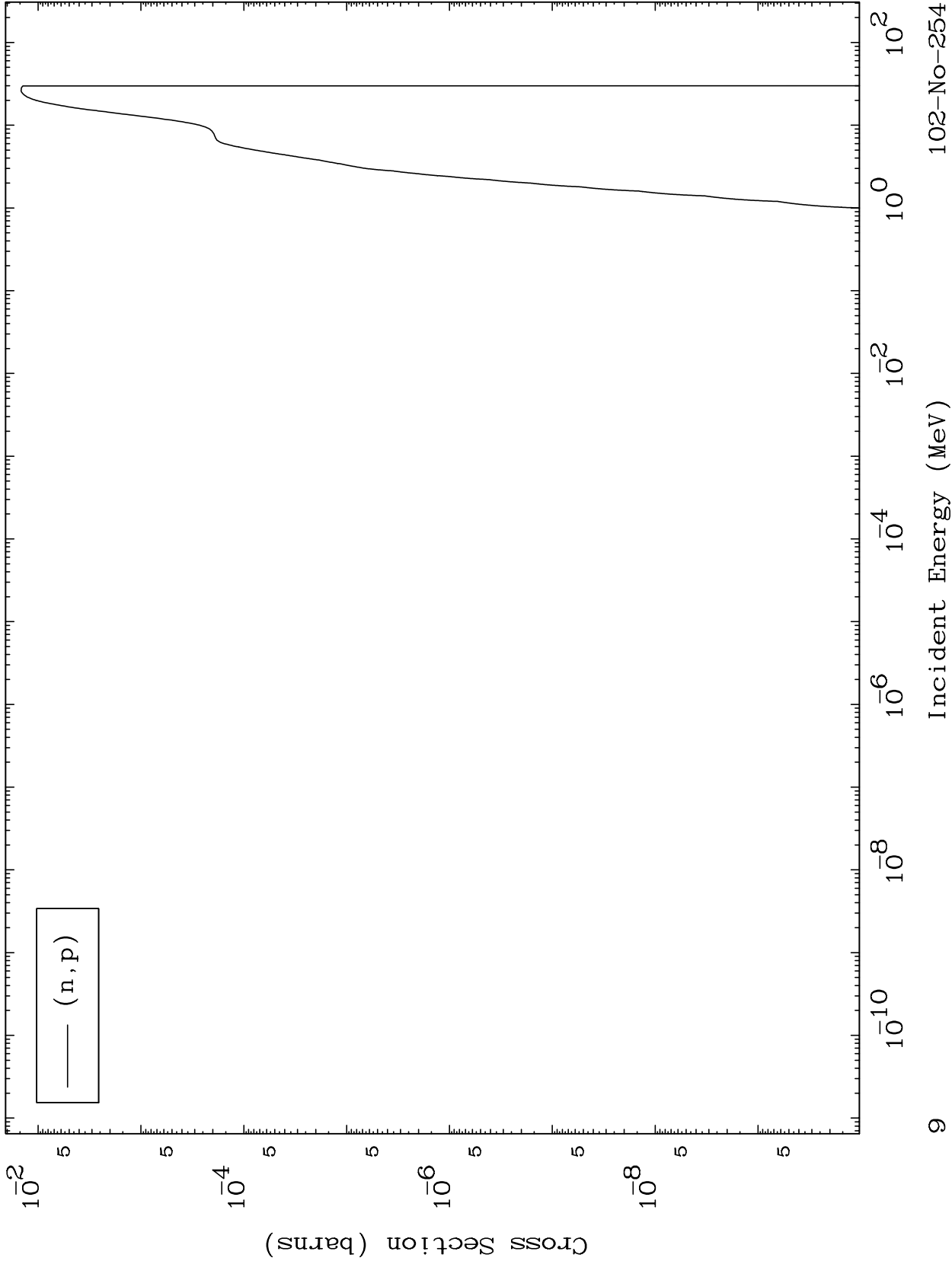
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

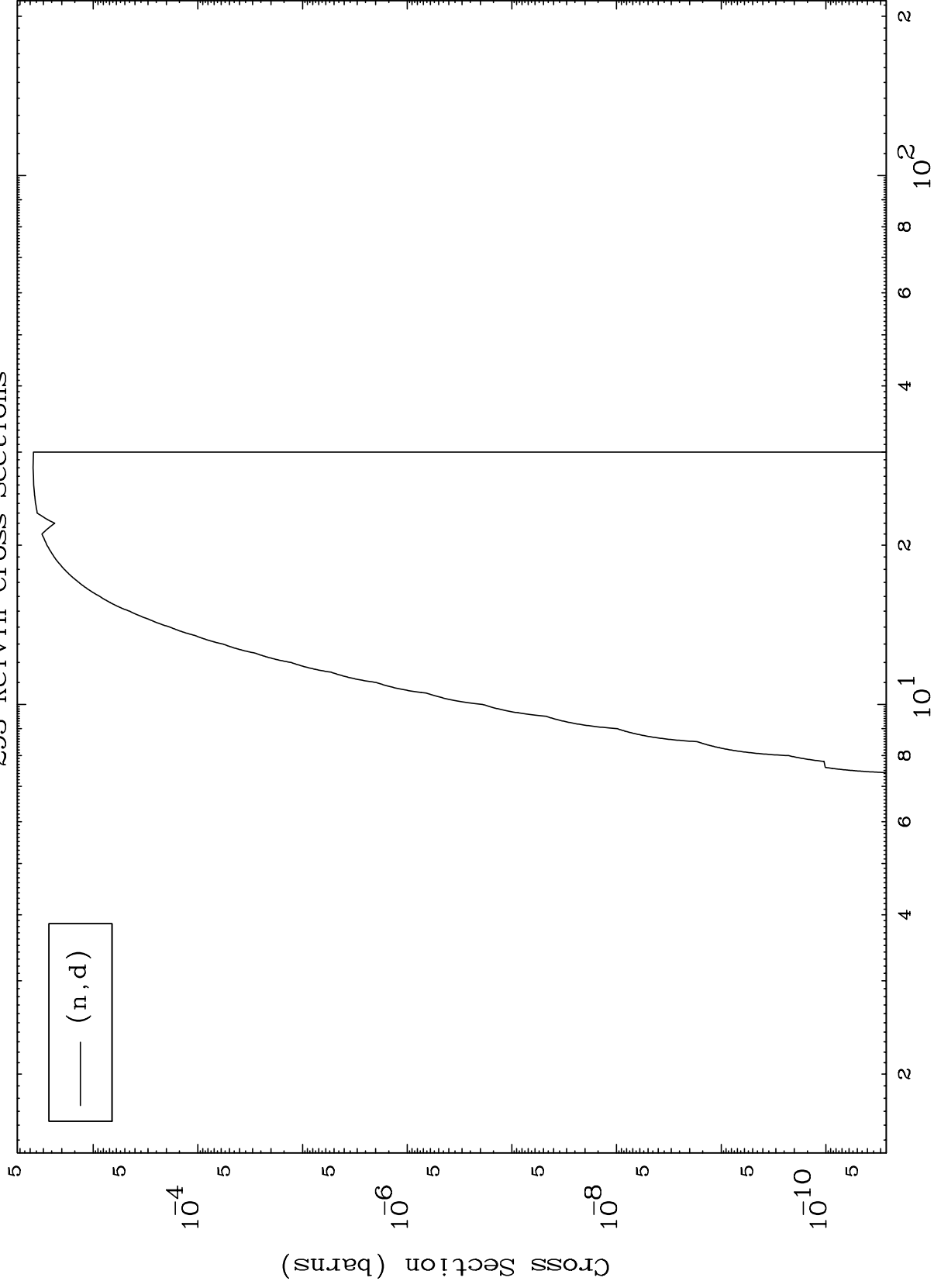
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(n,d) Levels
293 Kelvin Cross Sections

102-No-254



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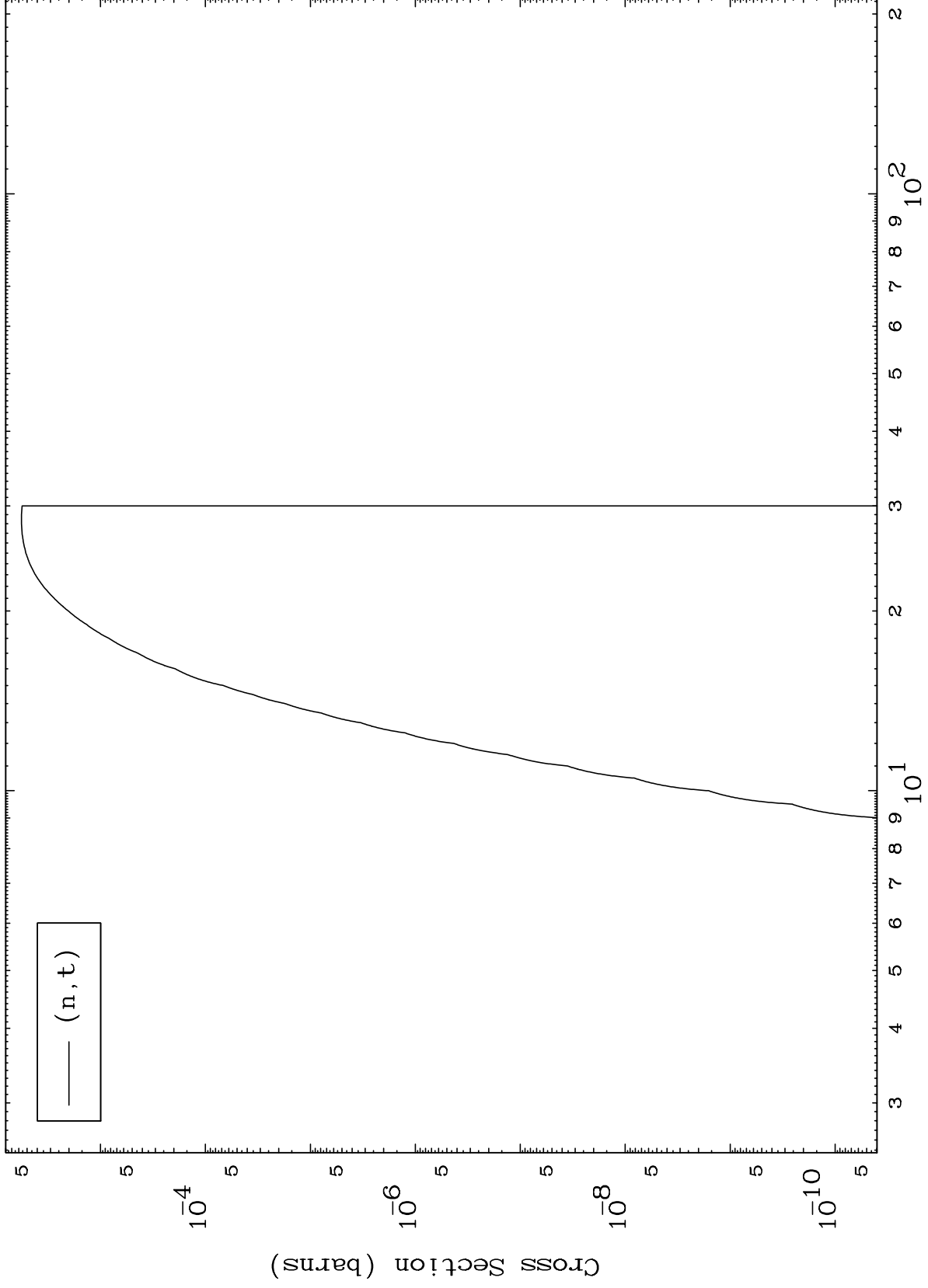
Incident Energy (MeV)

102-No-254

MAT 9972

(n,t) Levels
293 Kelvin Cross Sections

102-No-254



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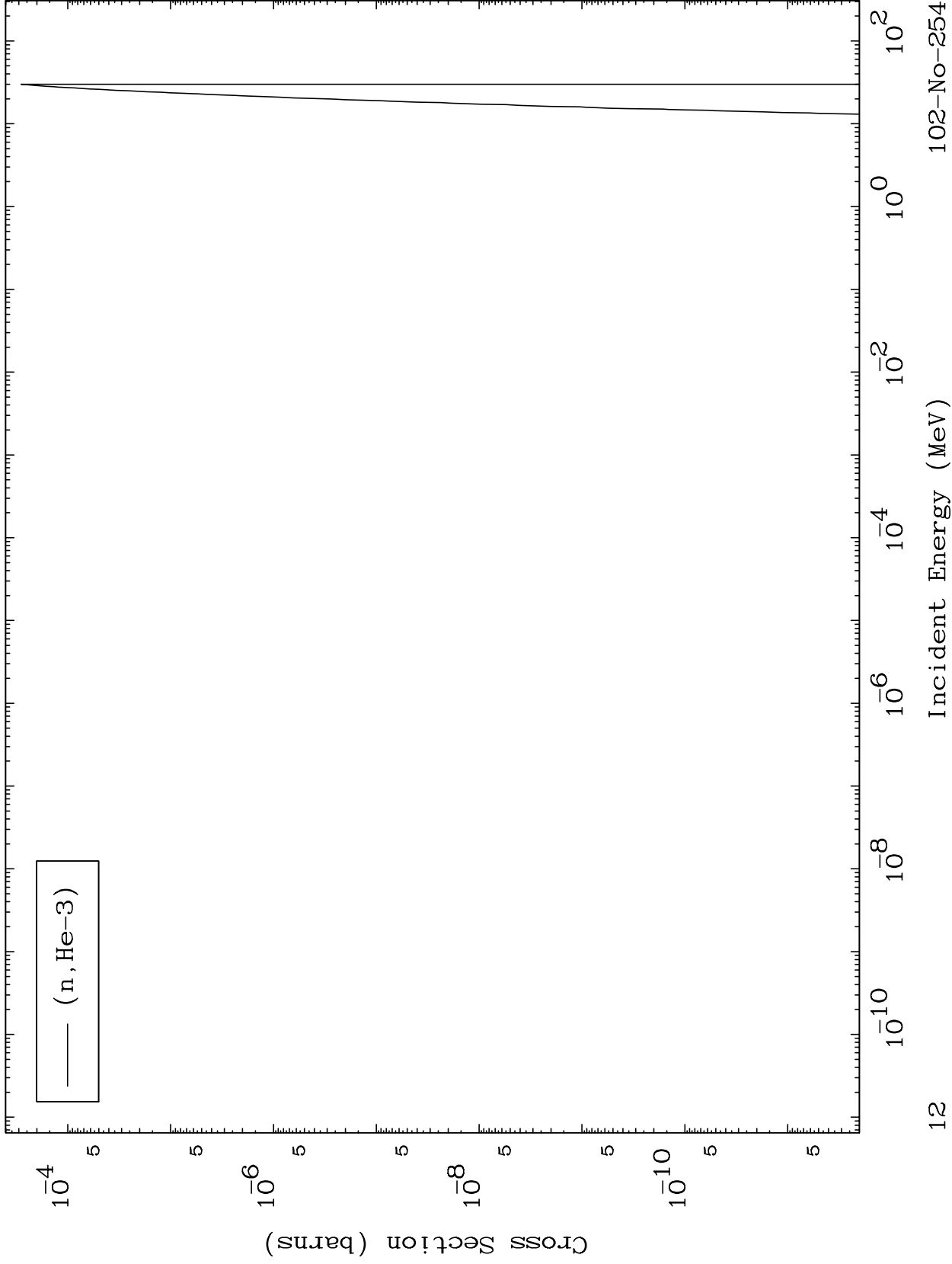
Incident Energy (MeV)

102-No-254

MAT 9972

(n,He3) Levels
293 Kelvin Cross Sections

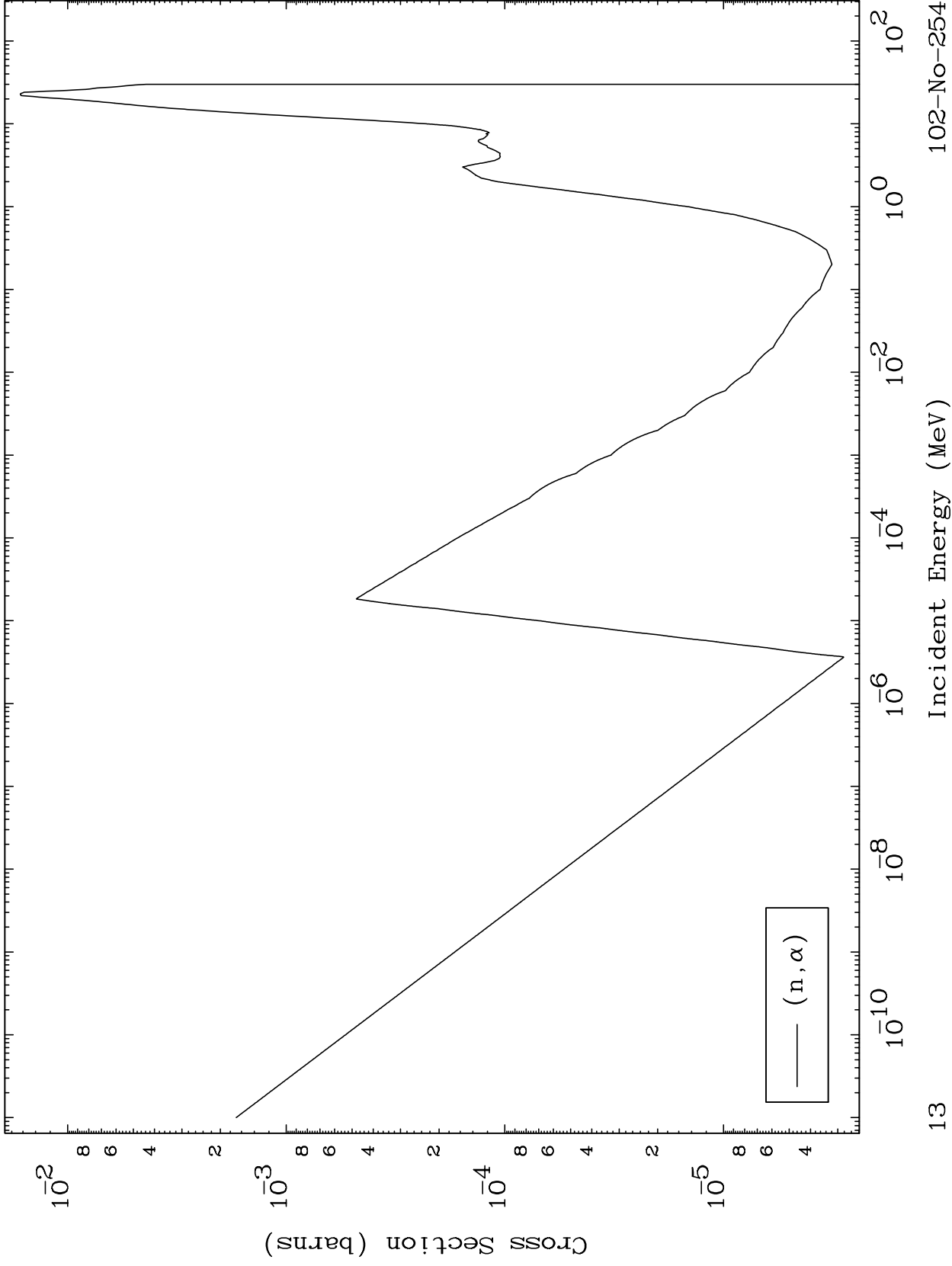
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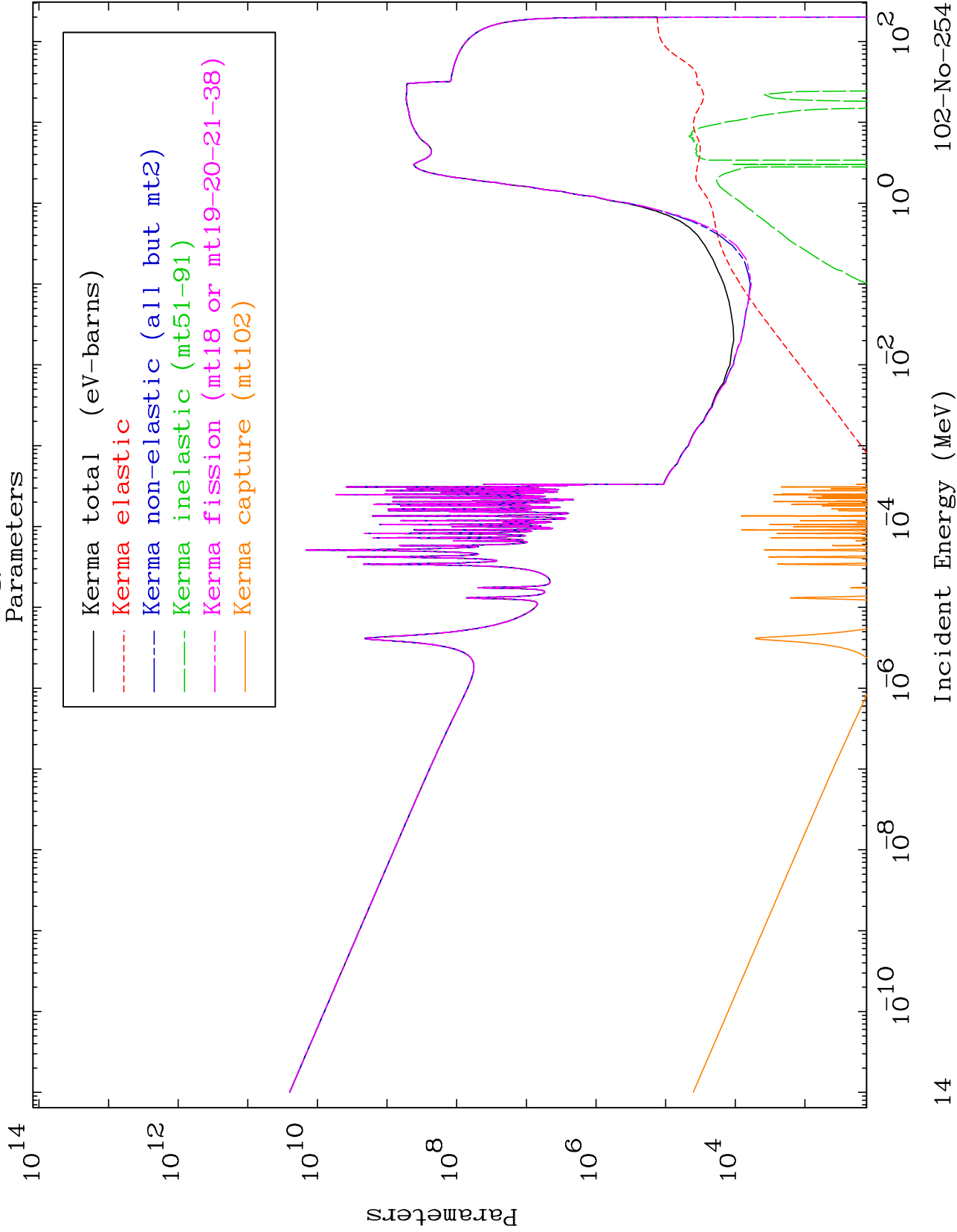


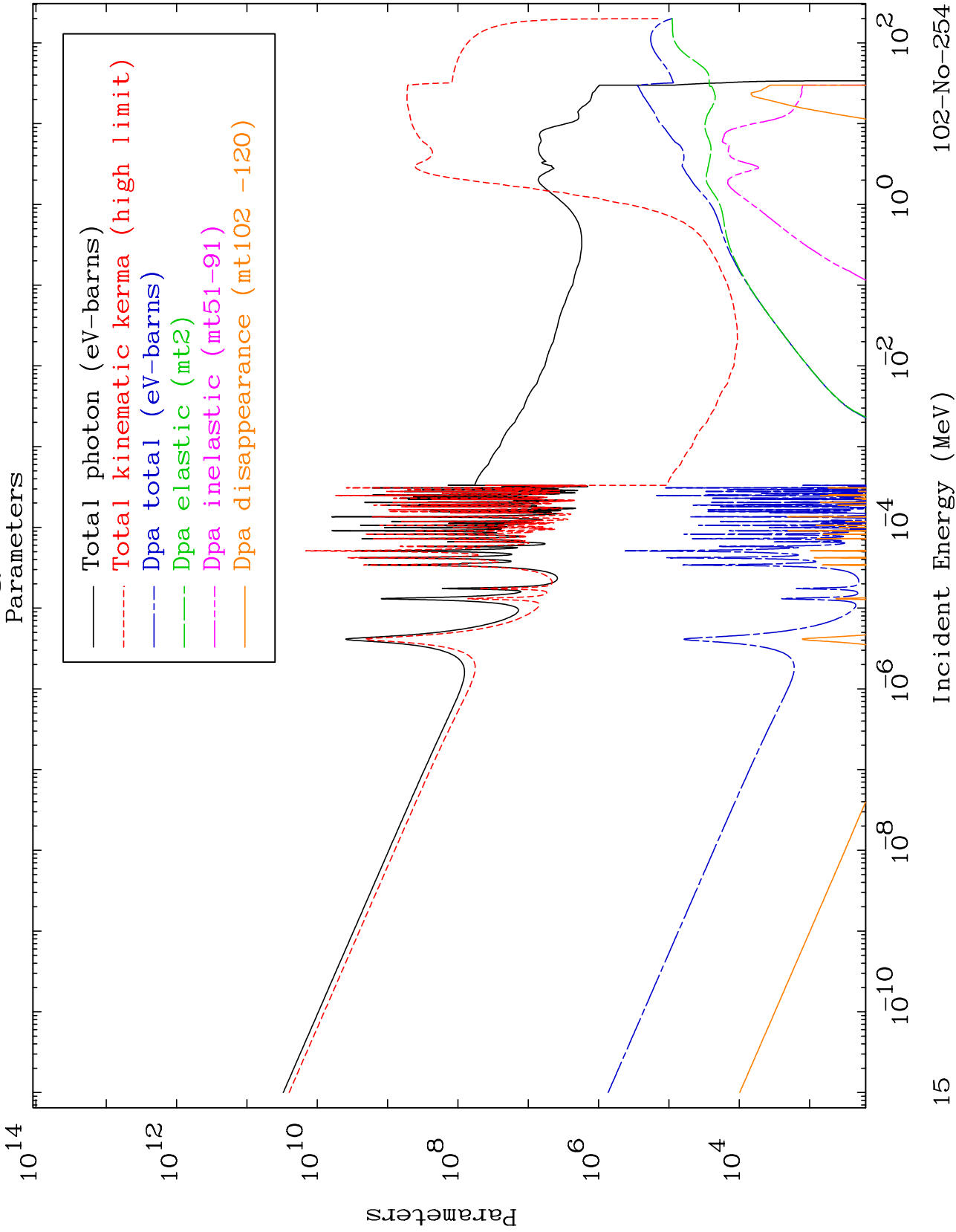
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(n, α) Levels
293 Kelvin Cross Sections

102-No-254



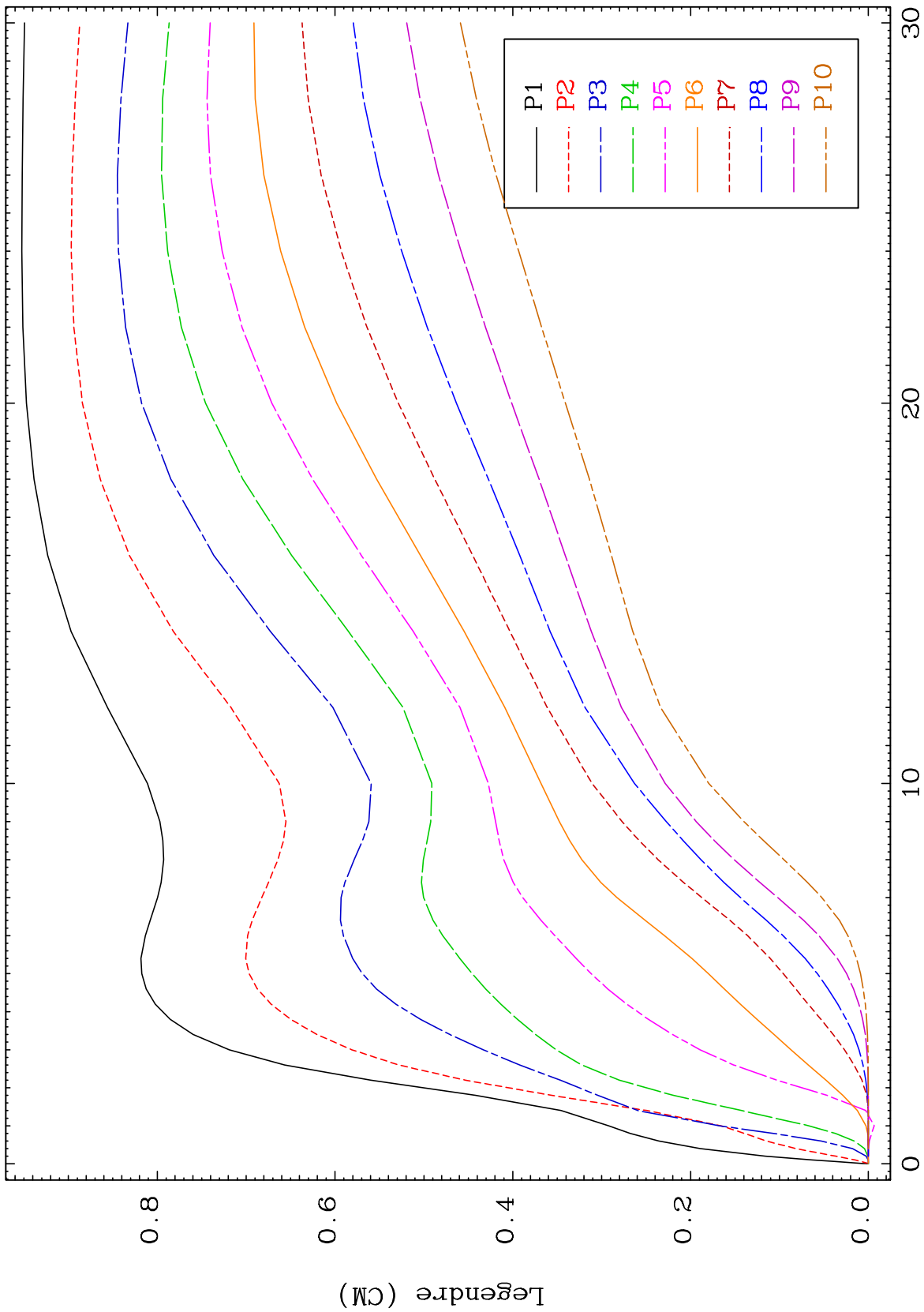




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Elastic Legendre Coefficients

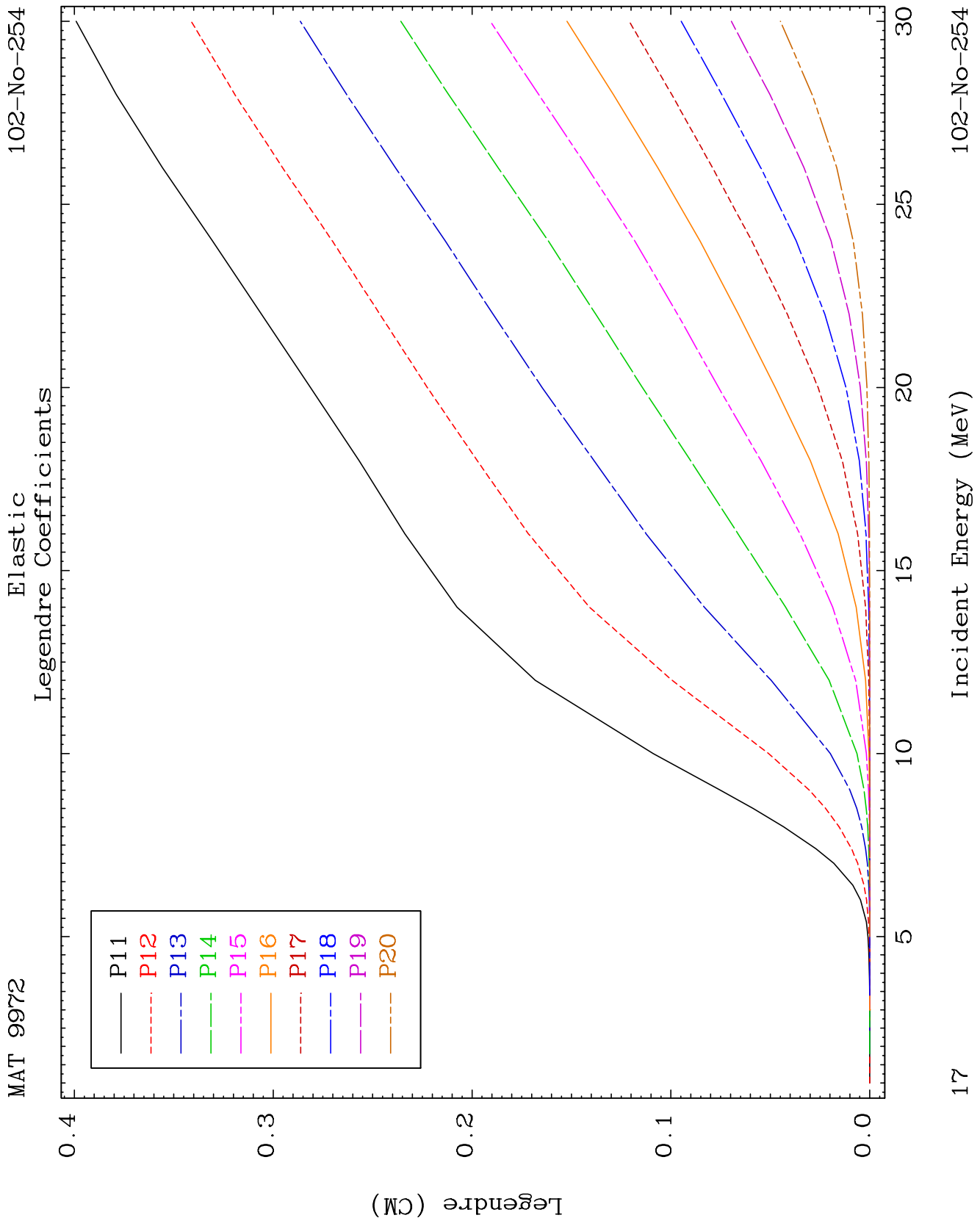
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Incident Energy (MeV)

102-No-254



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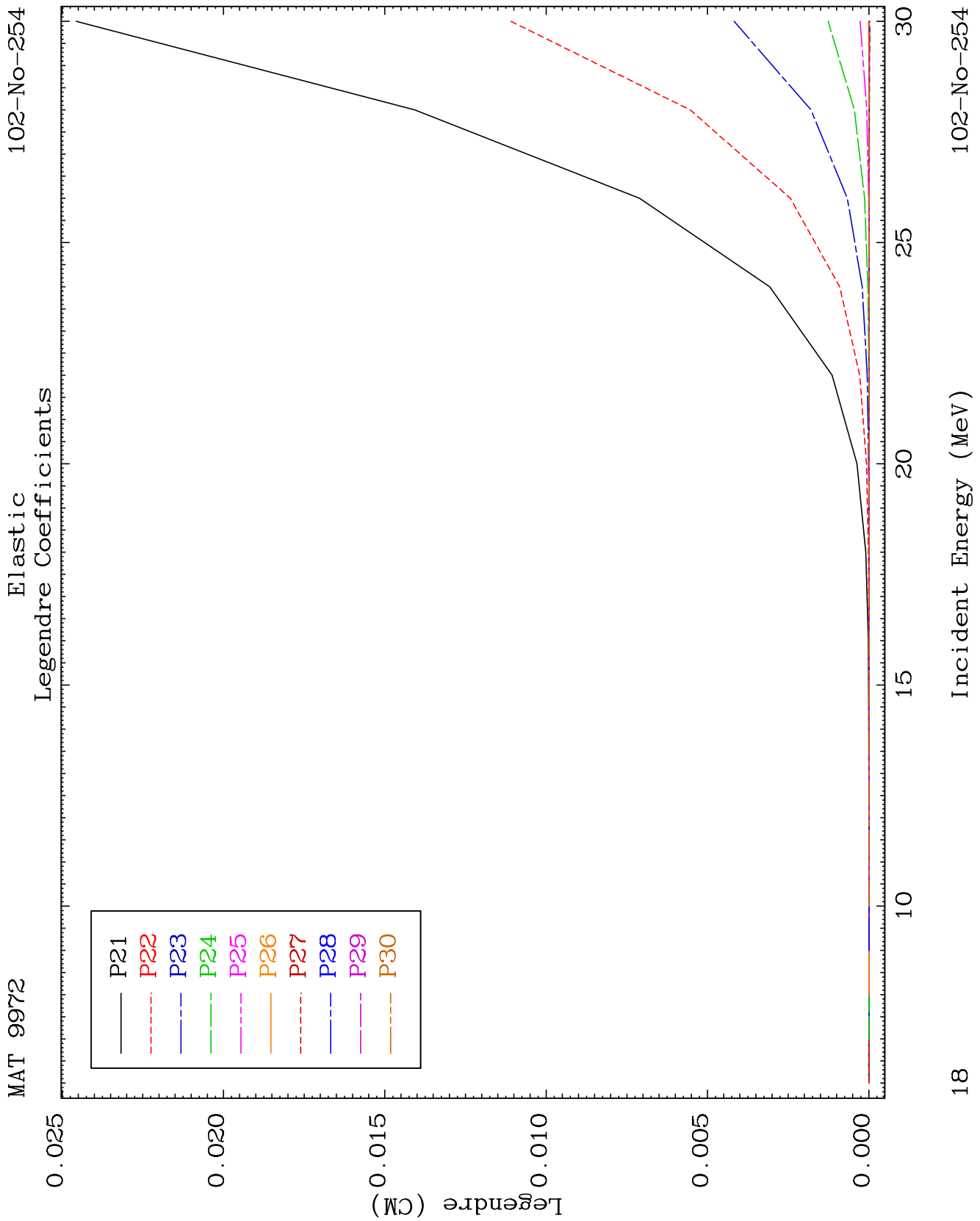
Elastic Legendre Coefficients

102-No-254

17

Incident Energy (MeV)

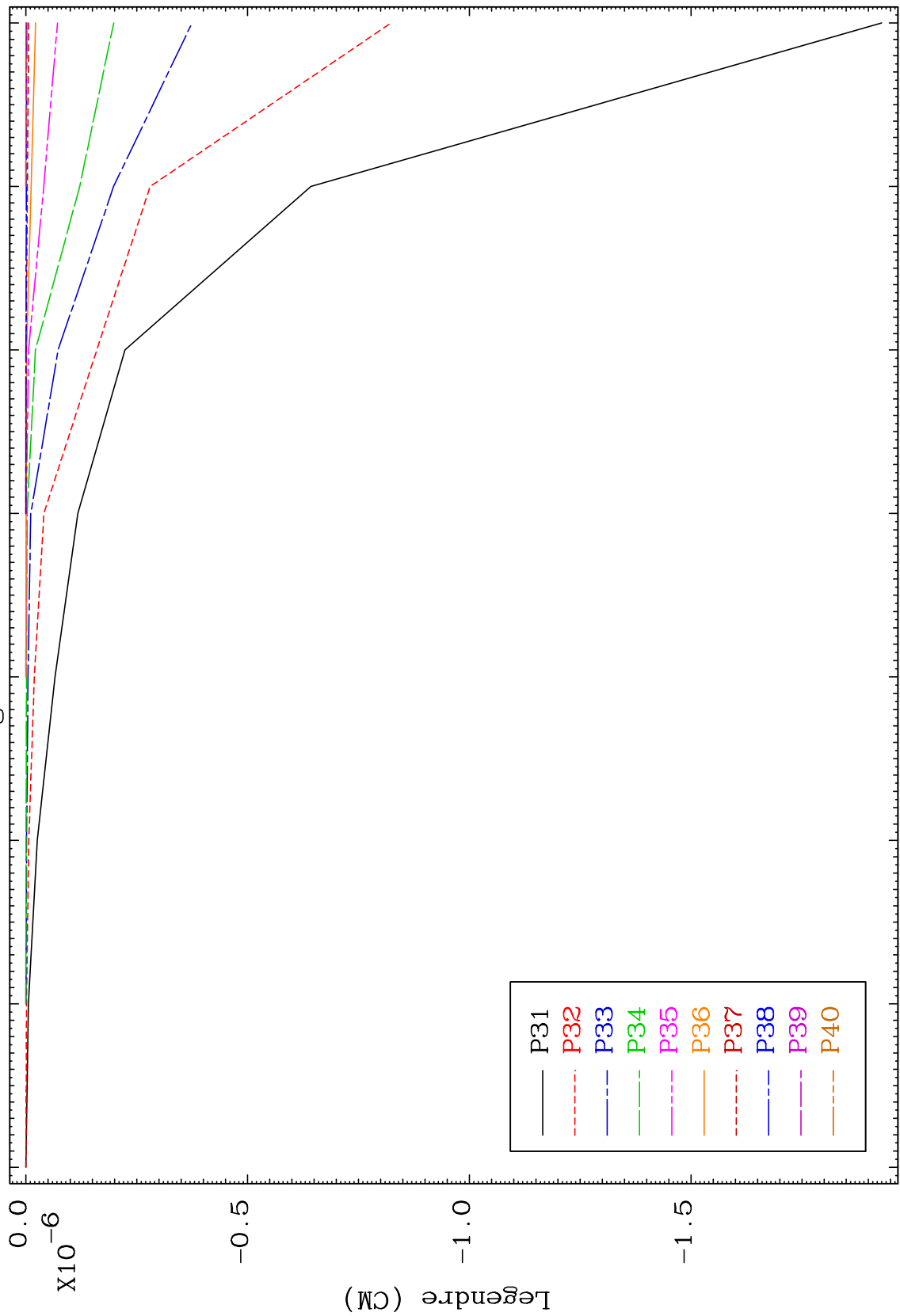
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Elastic Legendre Coefficients

102-No-254

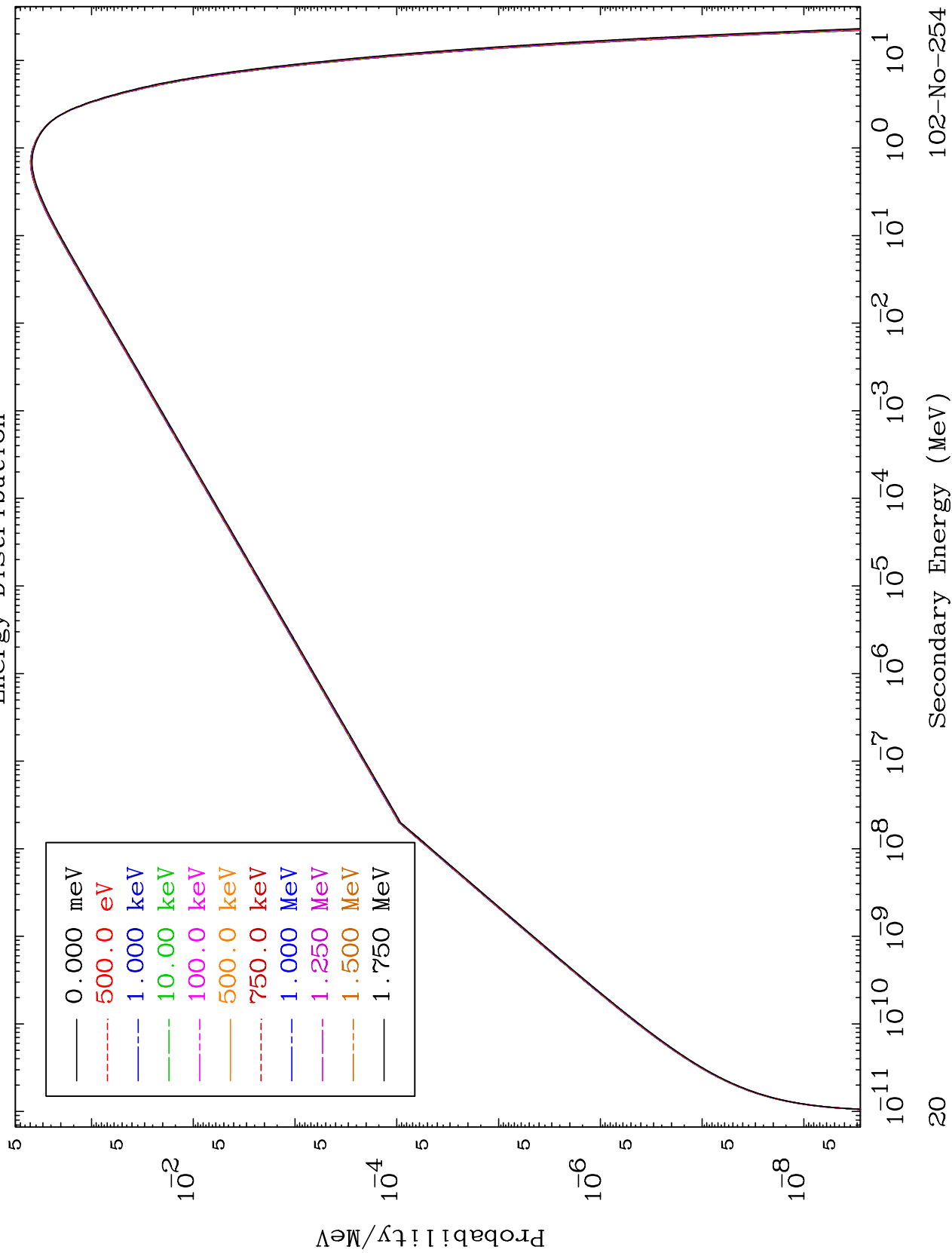


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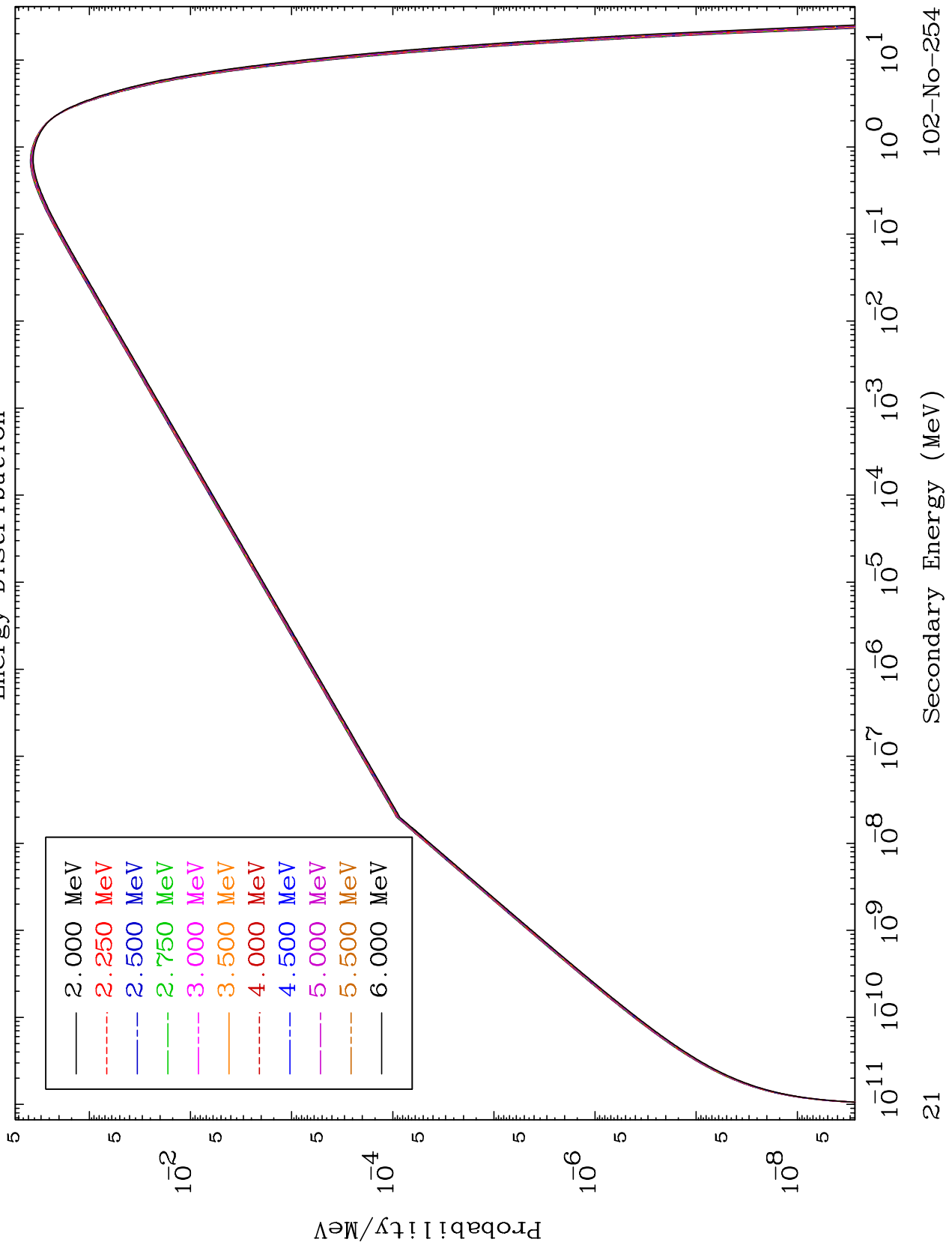
Incident Energy (MeV)

102-No-254

Fission Energy Distribution



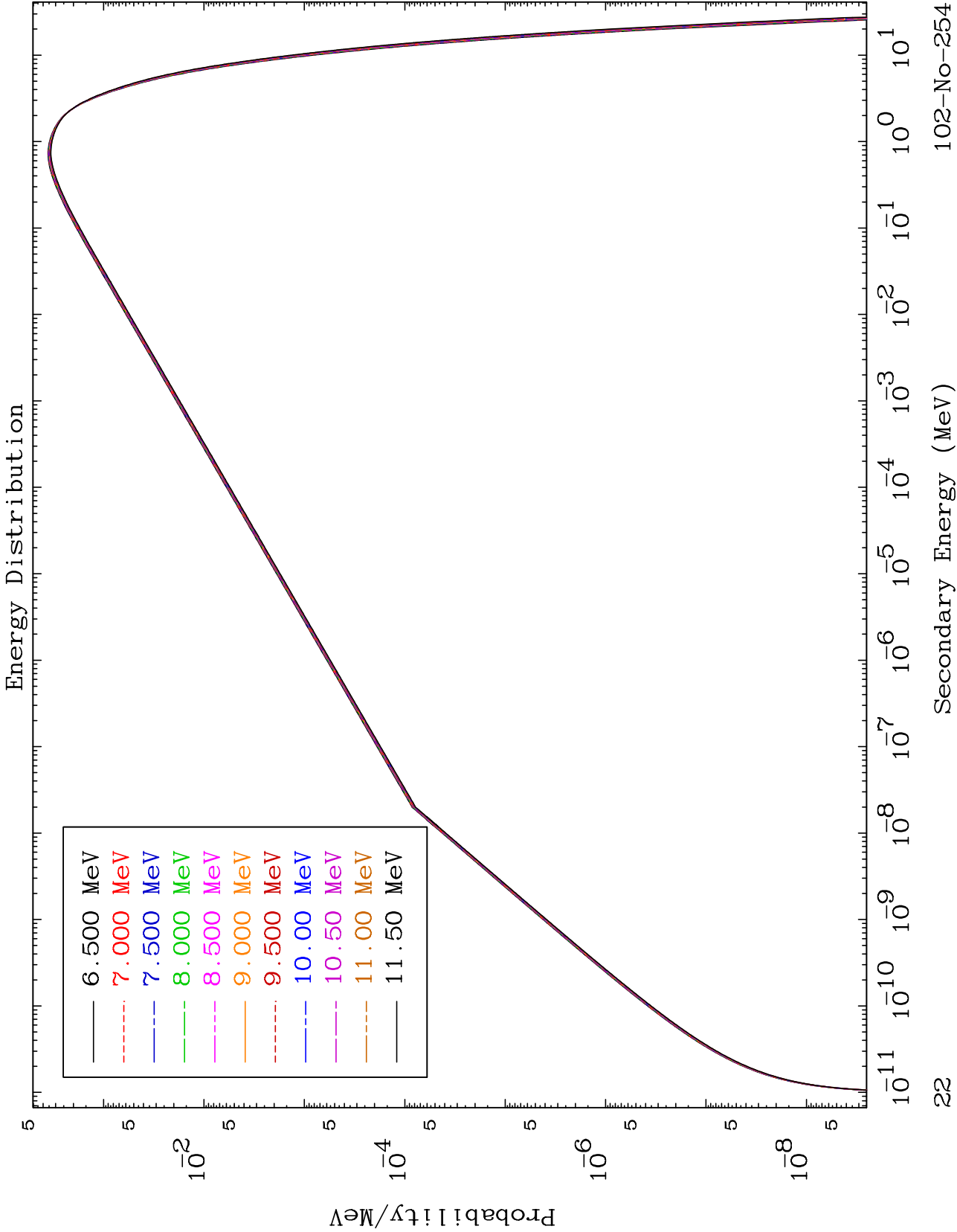
Fission Energy Distribution



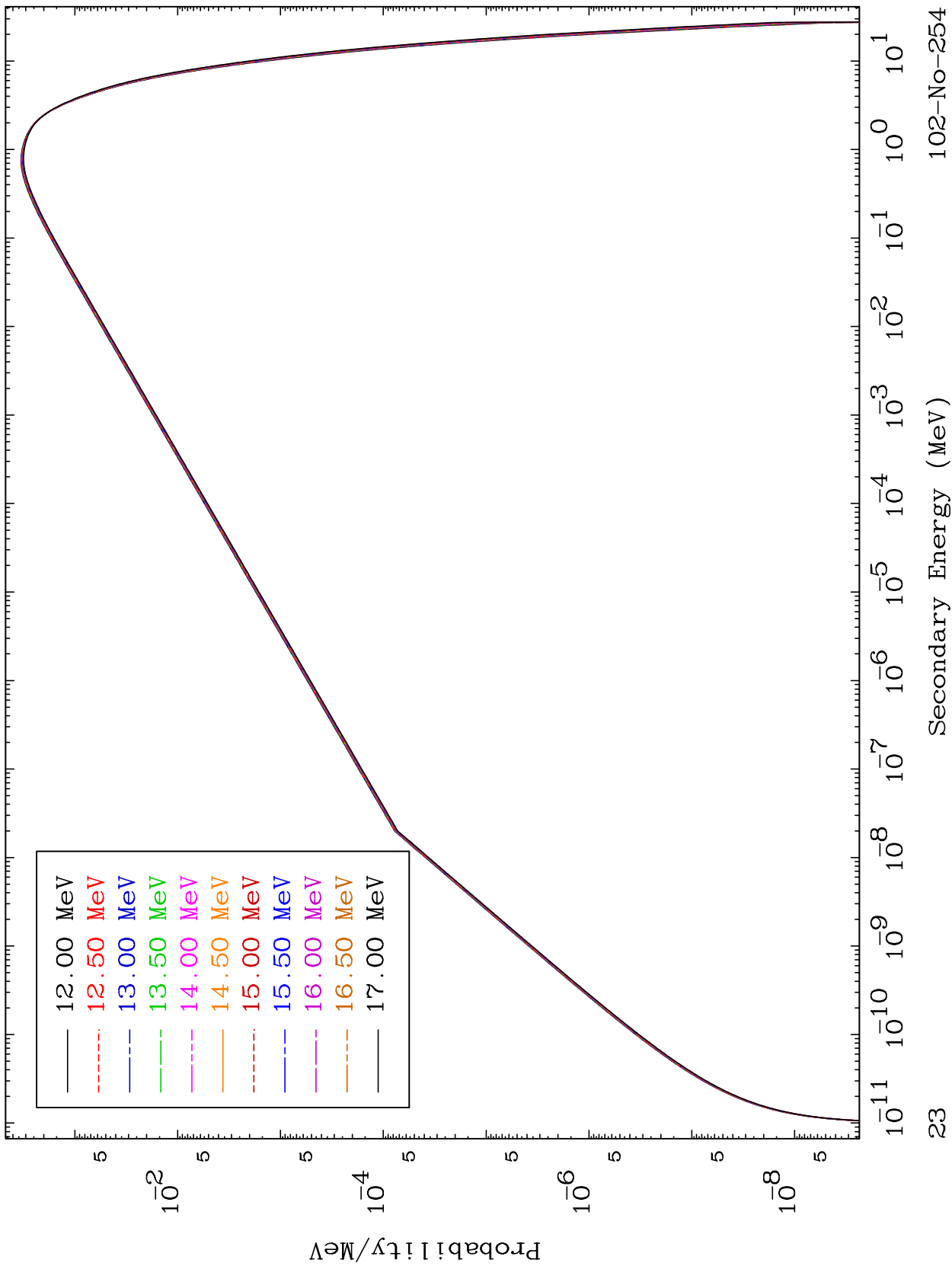
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Fission

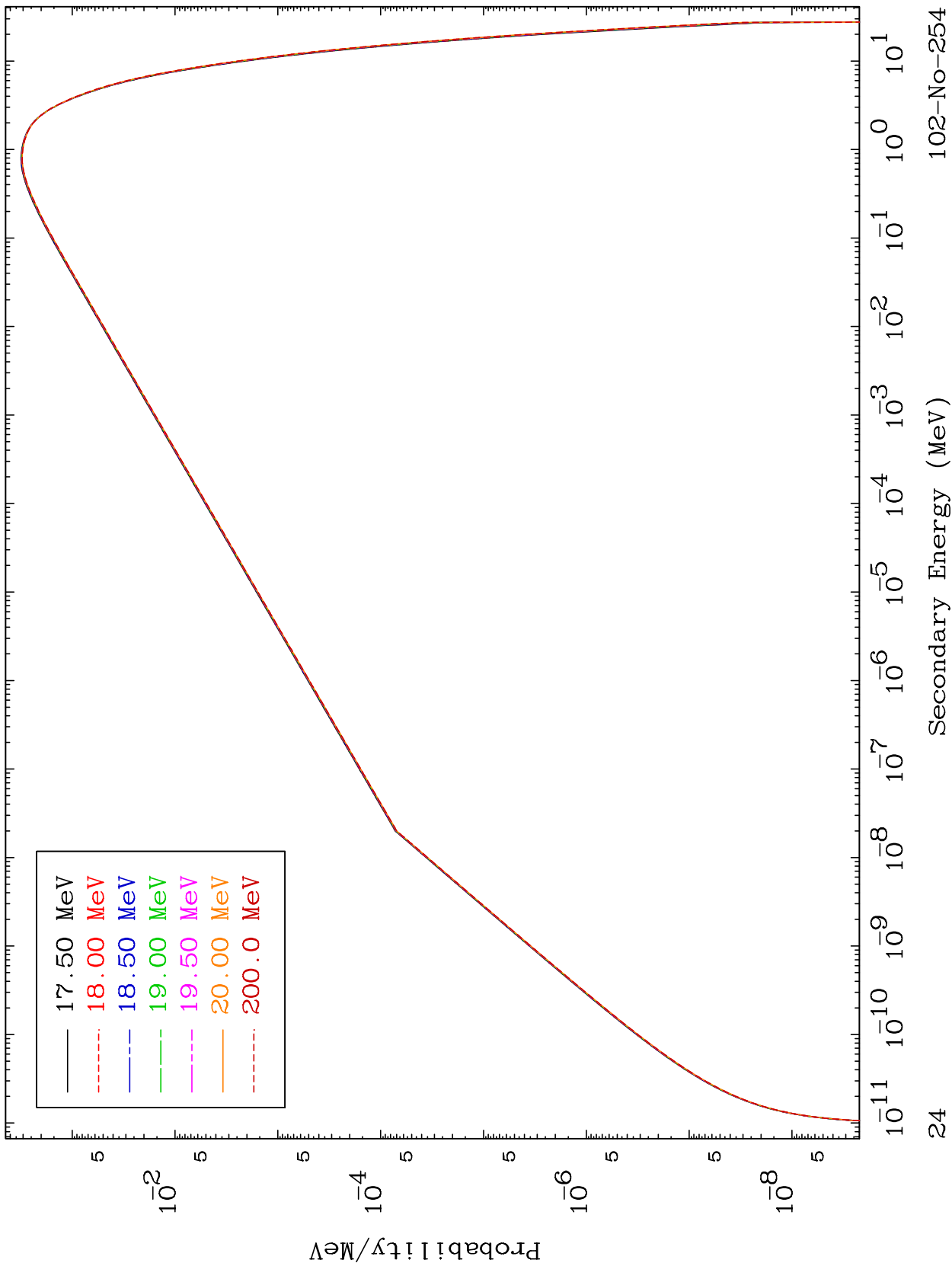
102-No-254



Fission Energy Distribution



Fission Energy Distribution

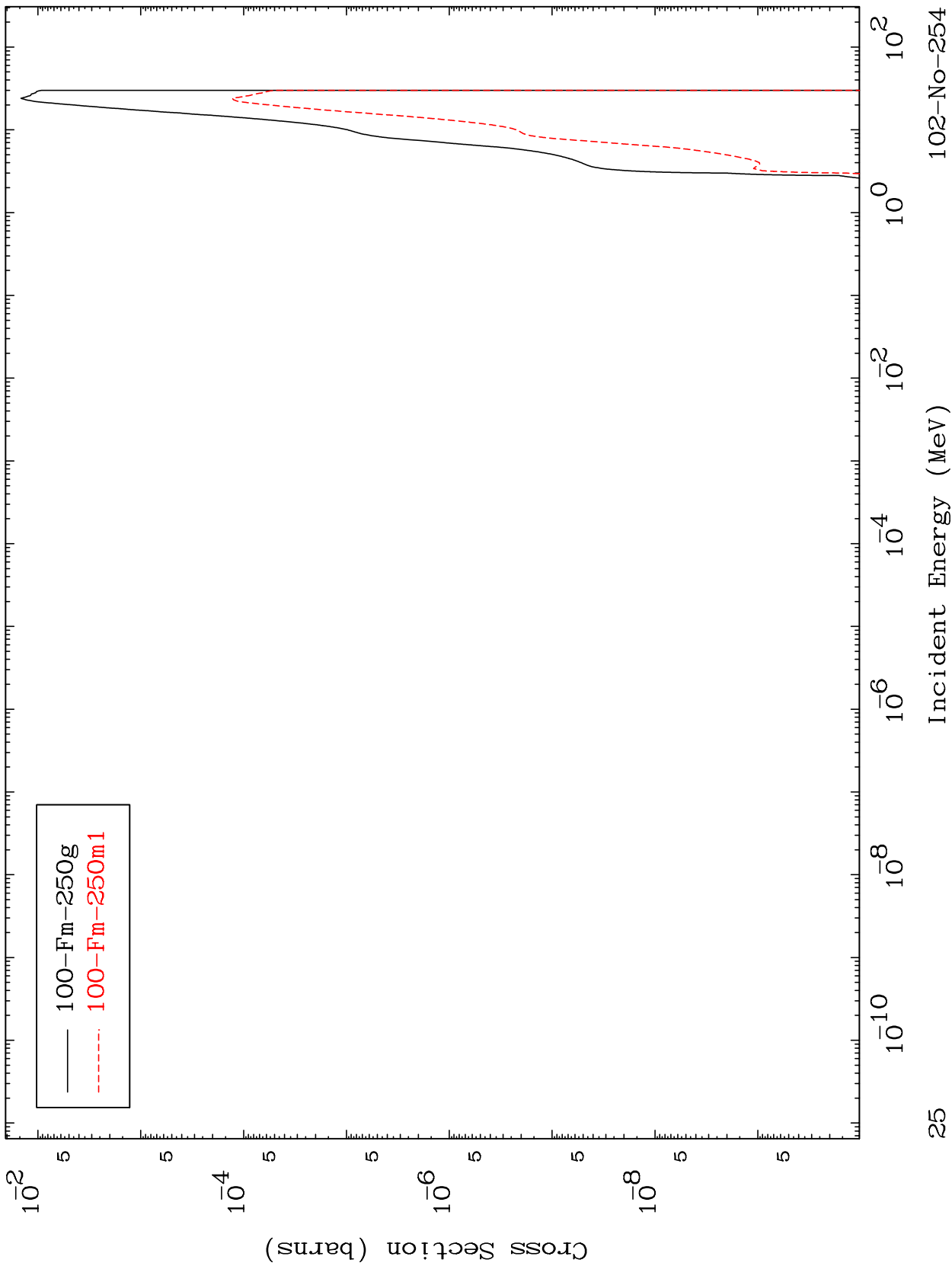


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$(n, n') \alpha$

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Radionuclide Production Cross Section

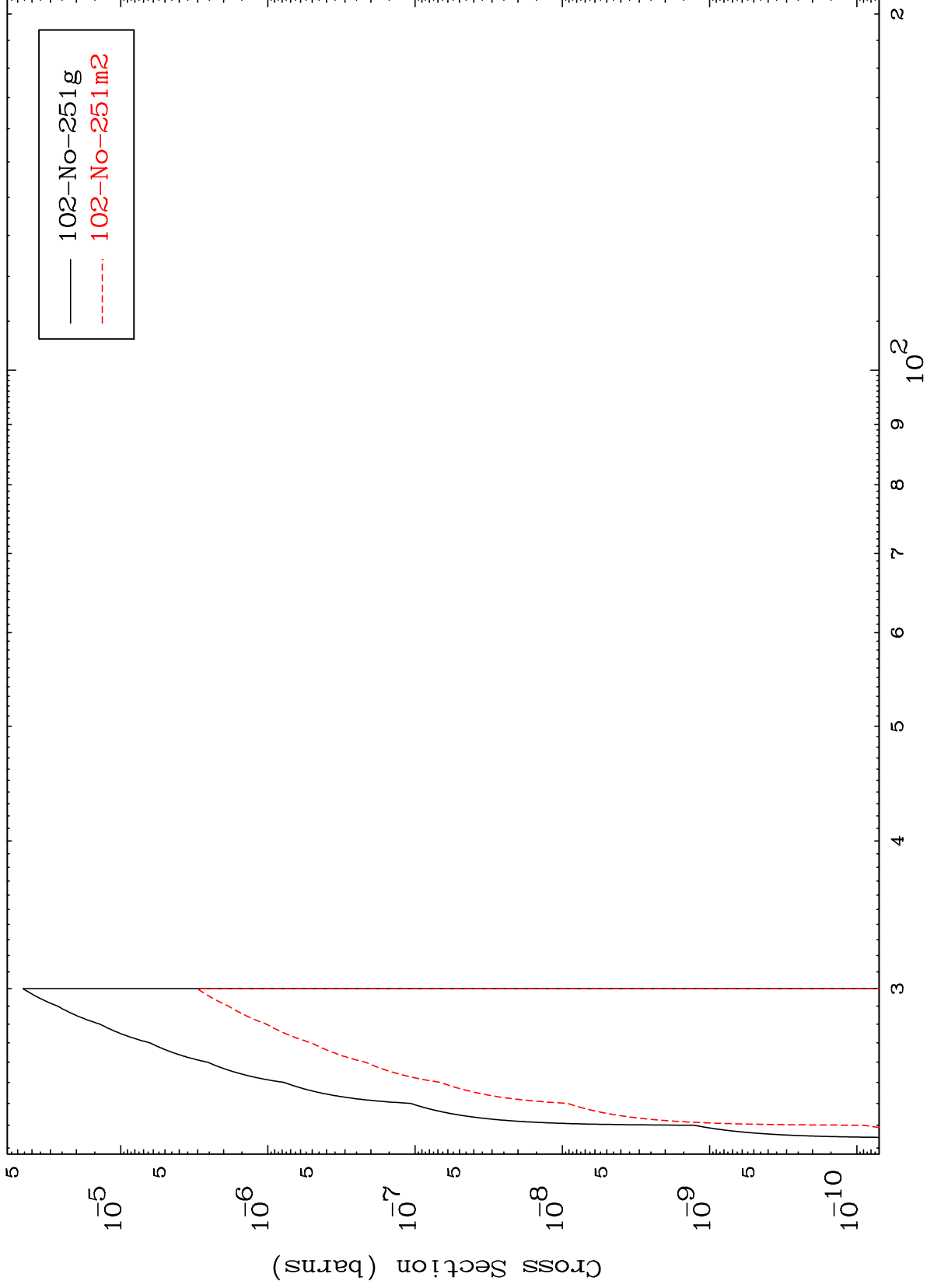


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(n,4n)

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Radionuclide Production Cross Section



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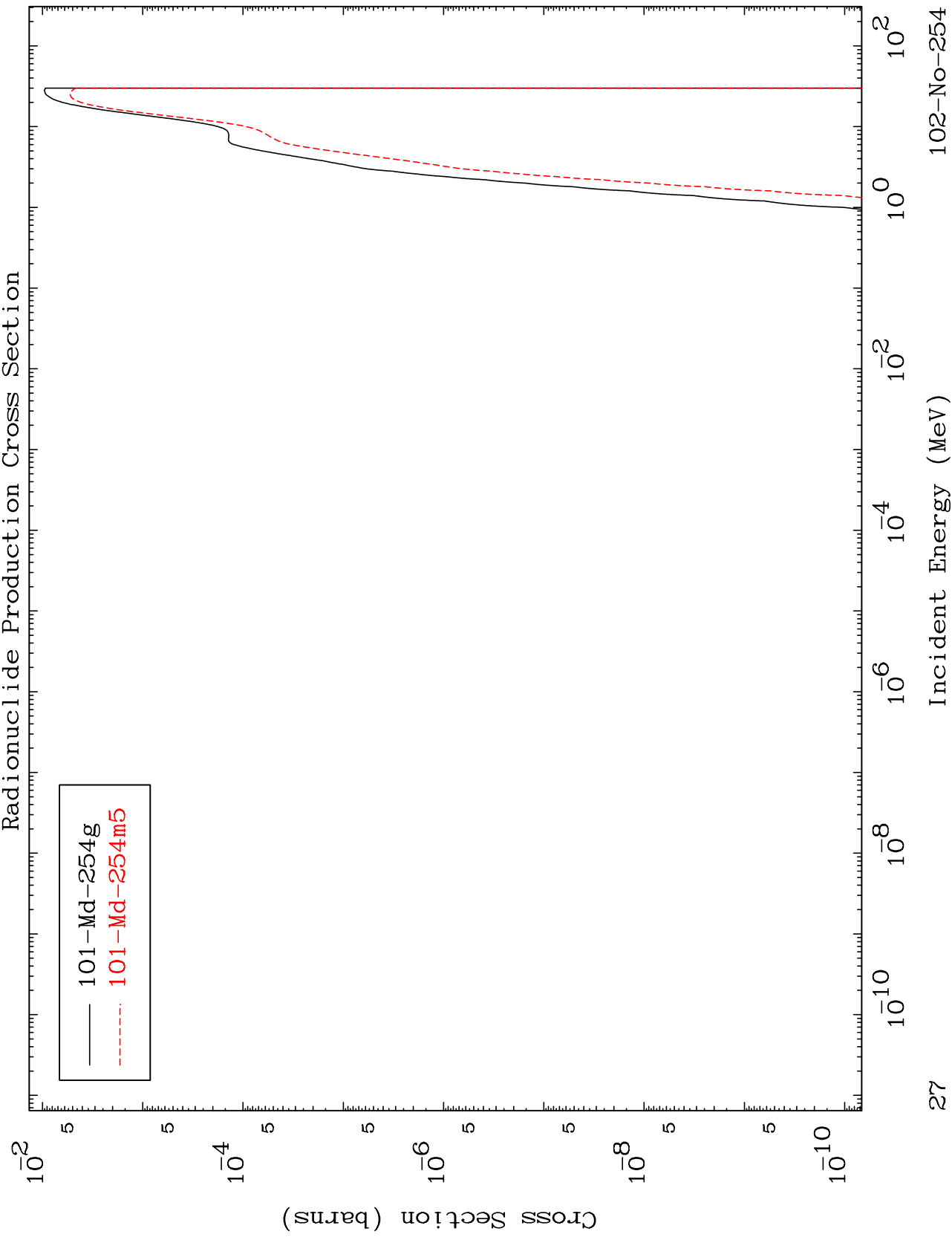
Incident Energy (MeV)

102-No-254

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(n,p)
Radionuclide Production Cross Section



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(n,p) α

102-No-254

Radionuclide Production Cross Section

